

Program EVALPLOT
(Version 2021-1)

by

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(Present Contact Information)

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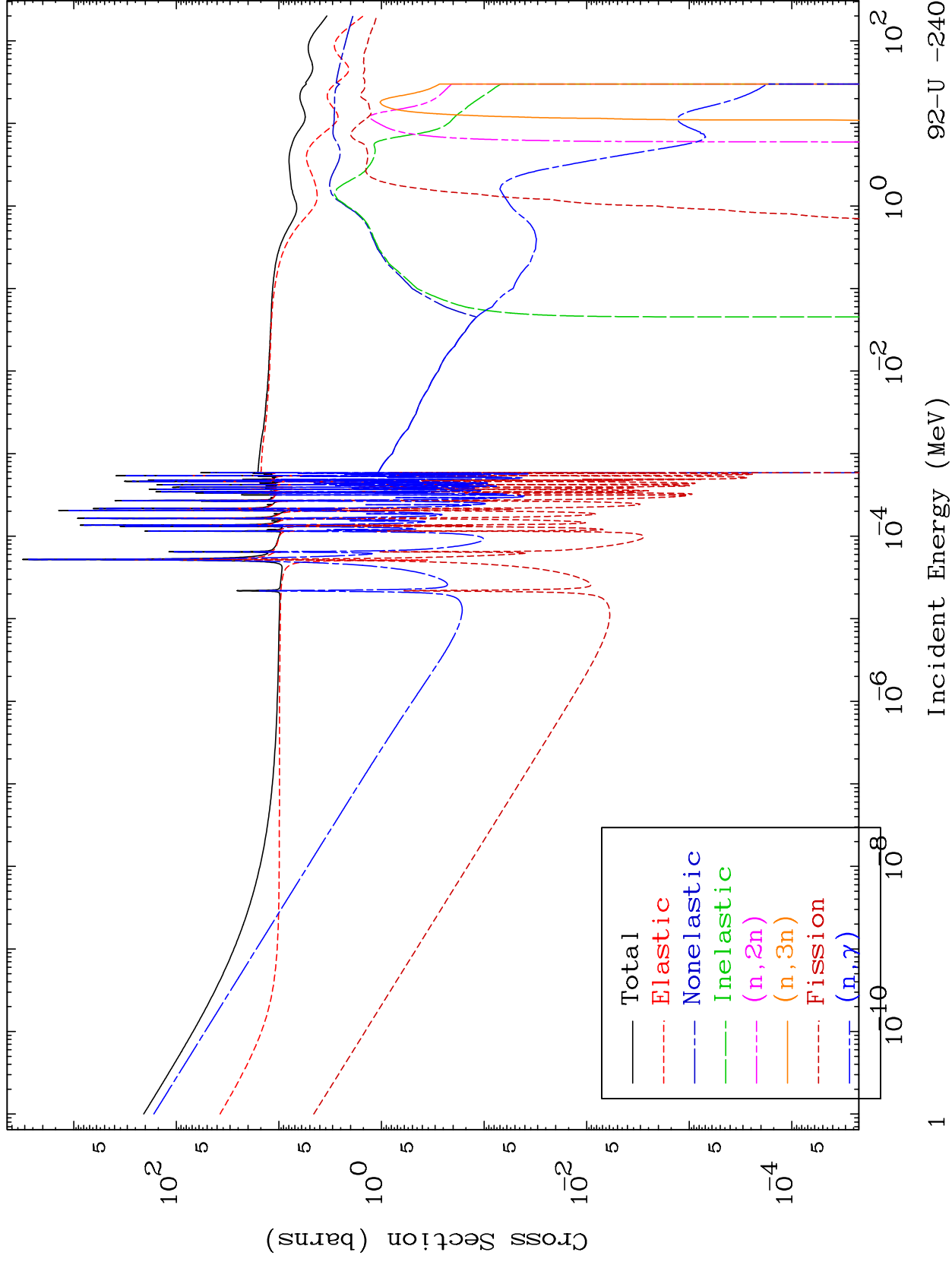
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

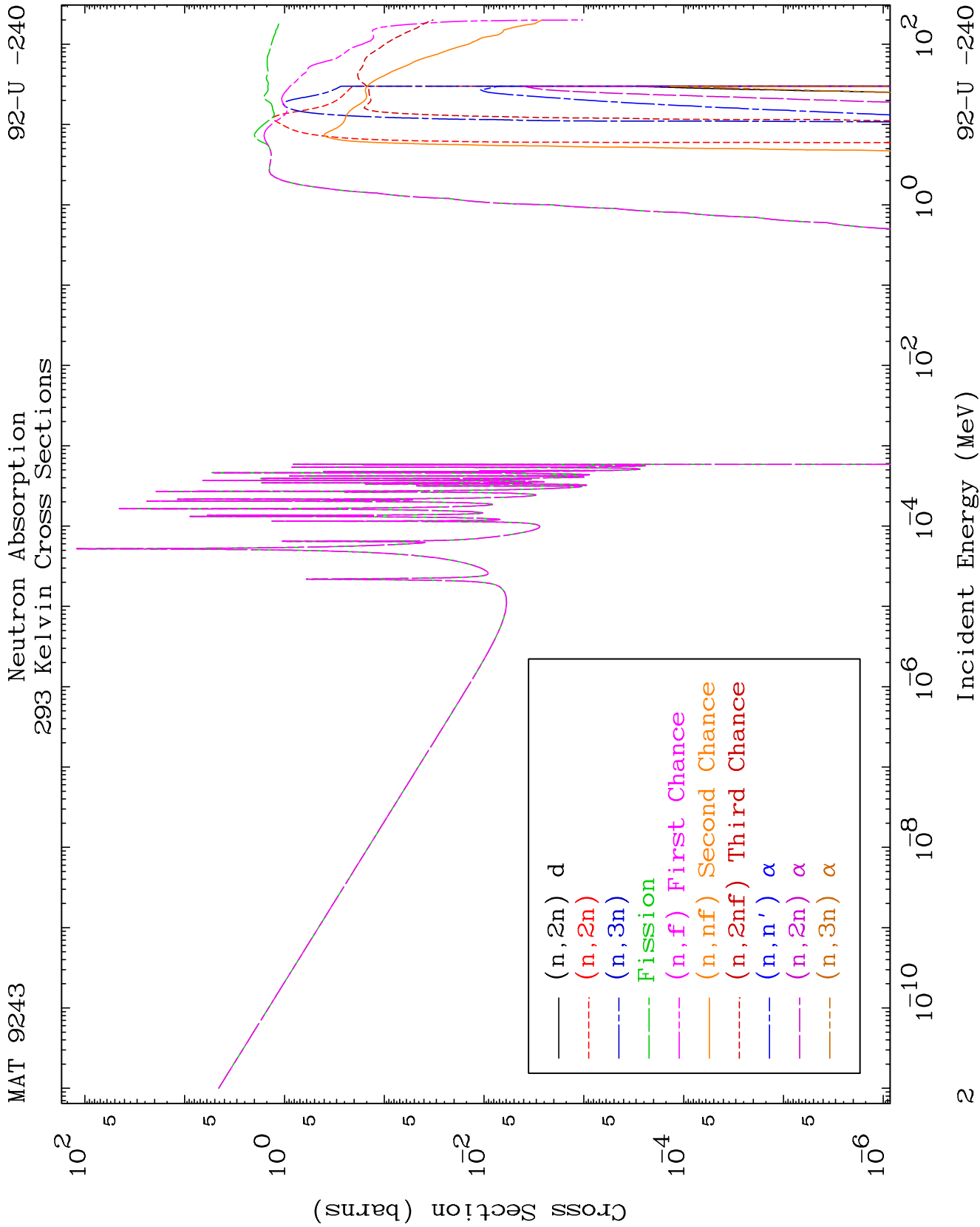
Press Mouse Button to Start

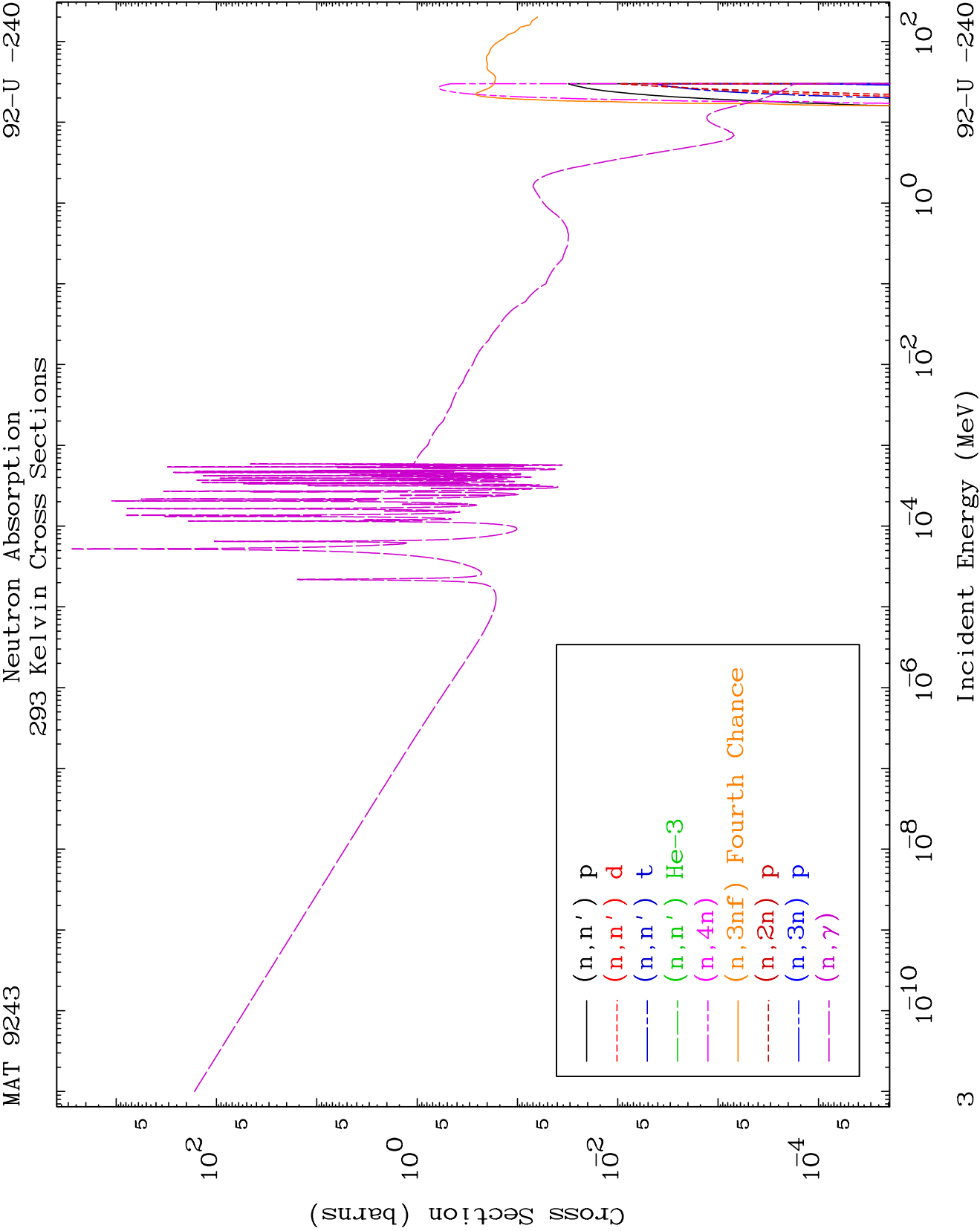
MAT 9243

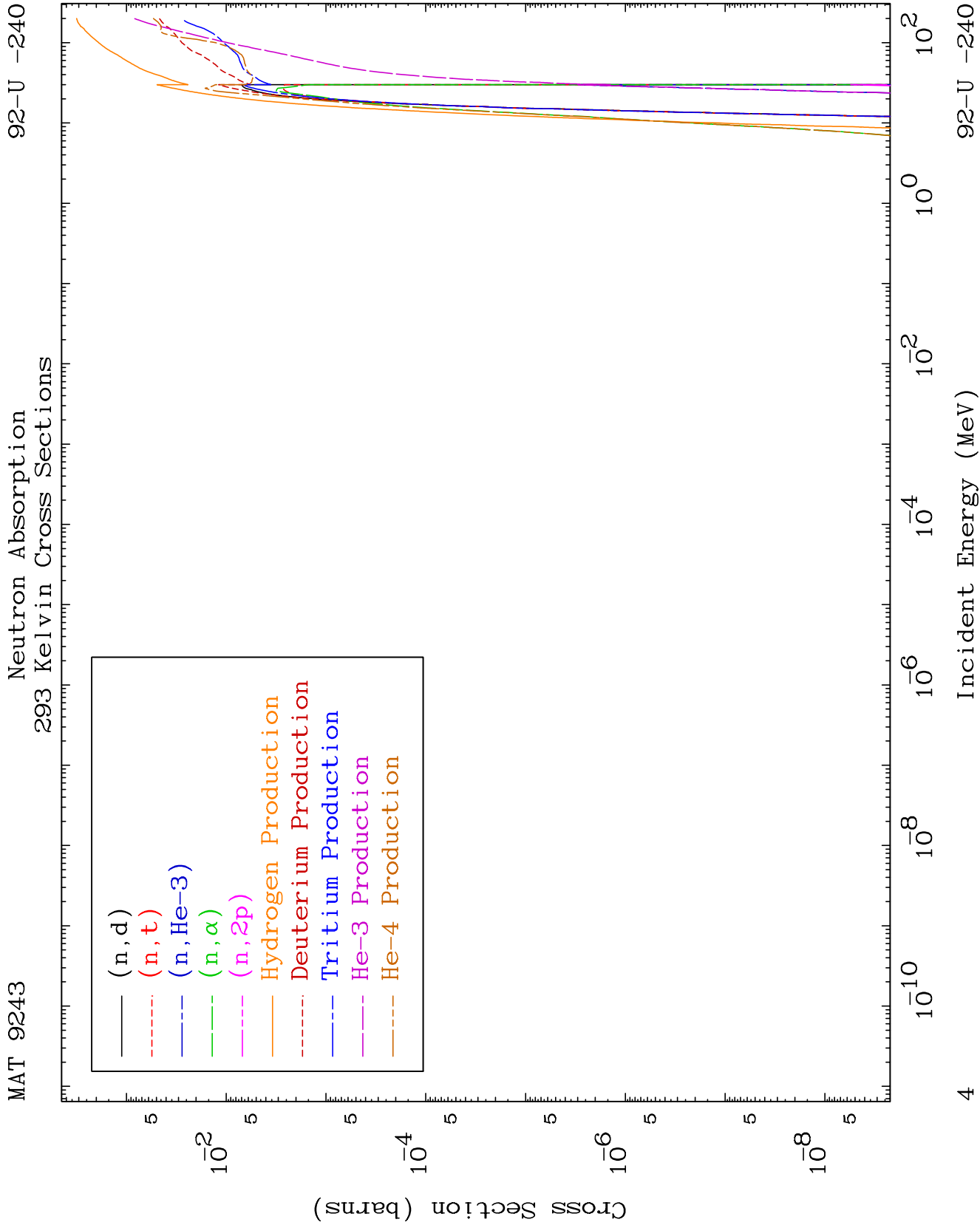
Neutron Major
293 Kelvin Cross Sections

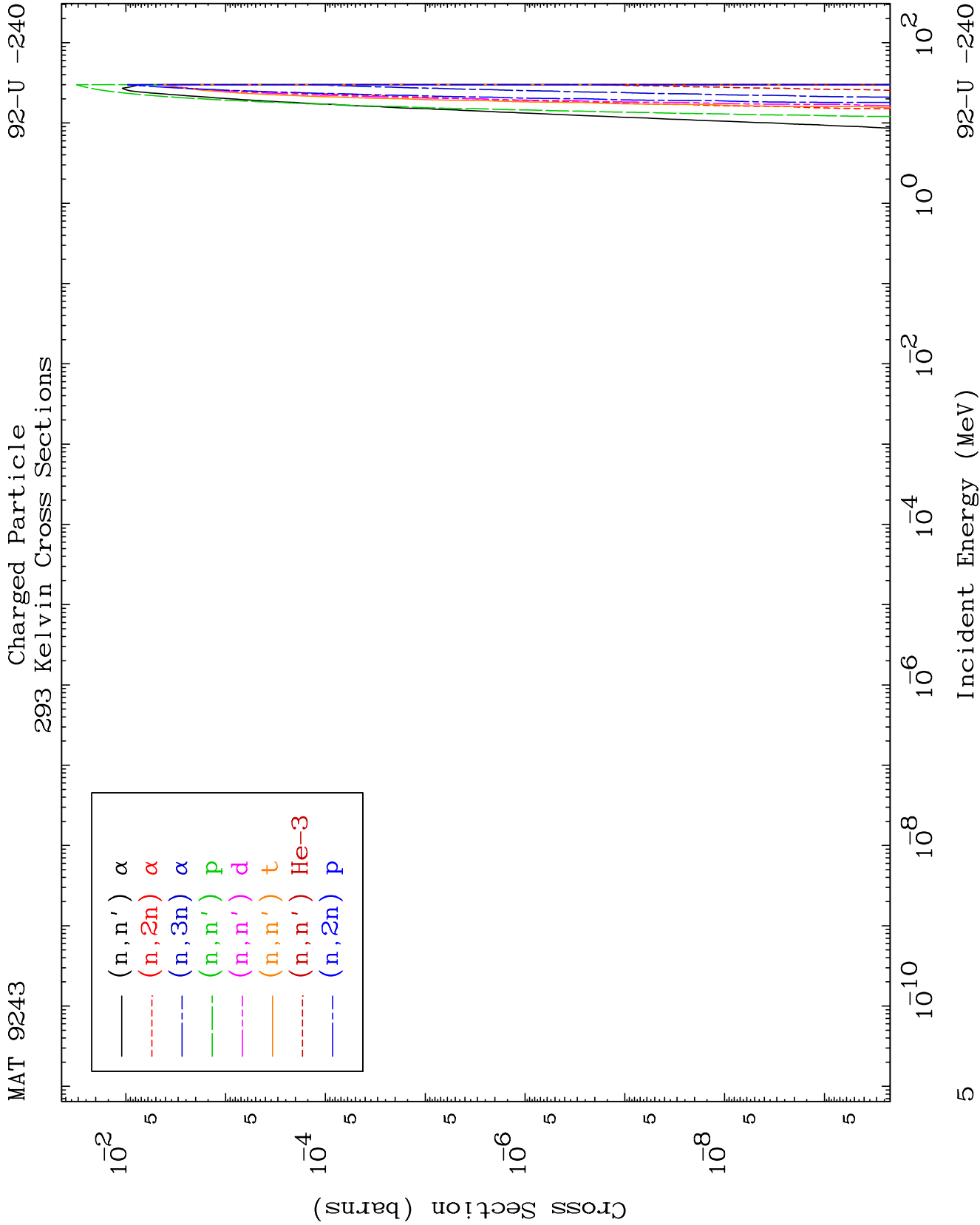
92-U -240

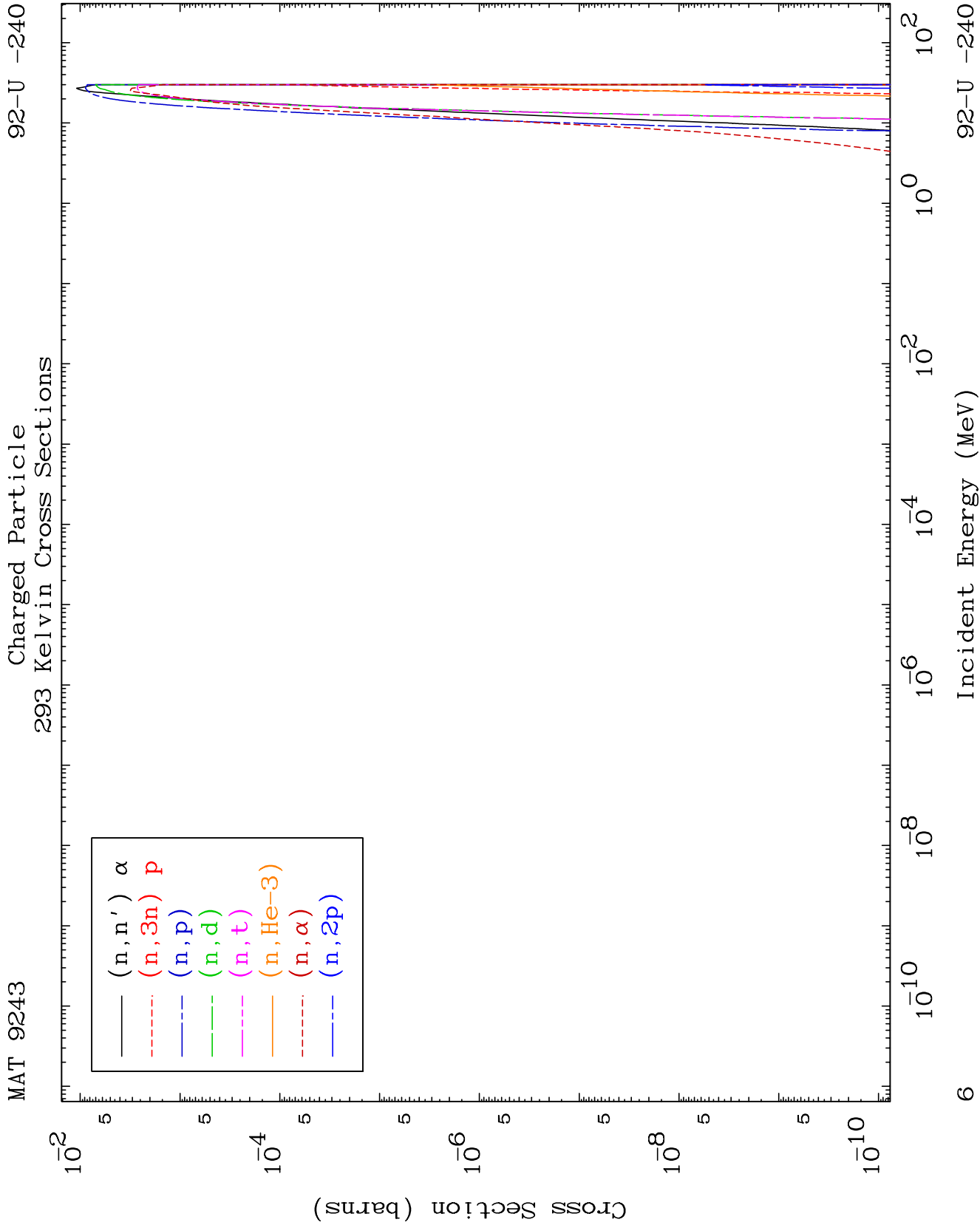


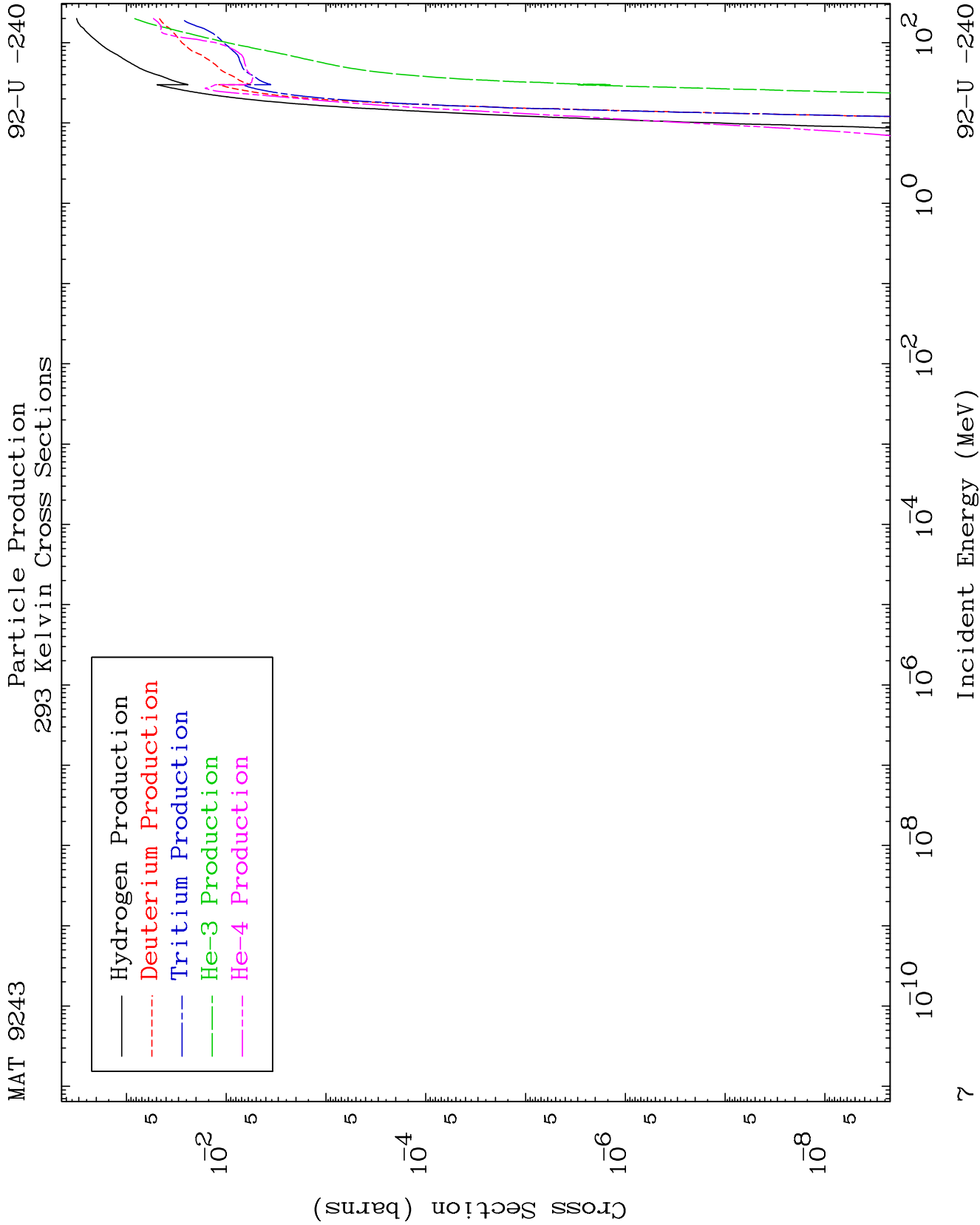


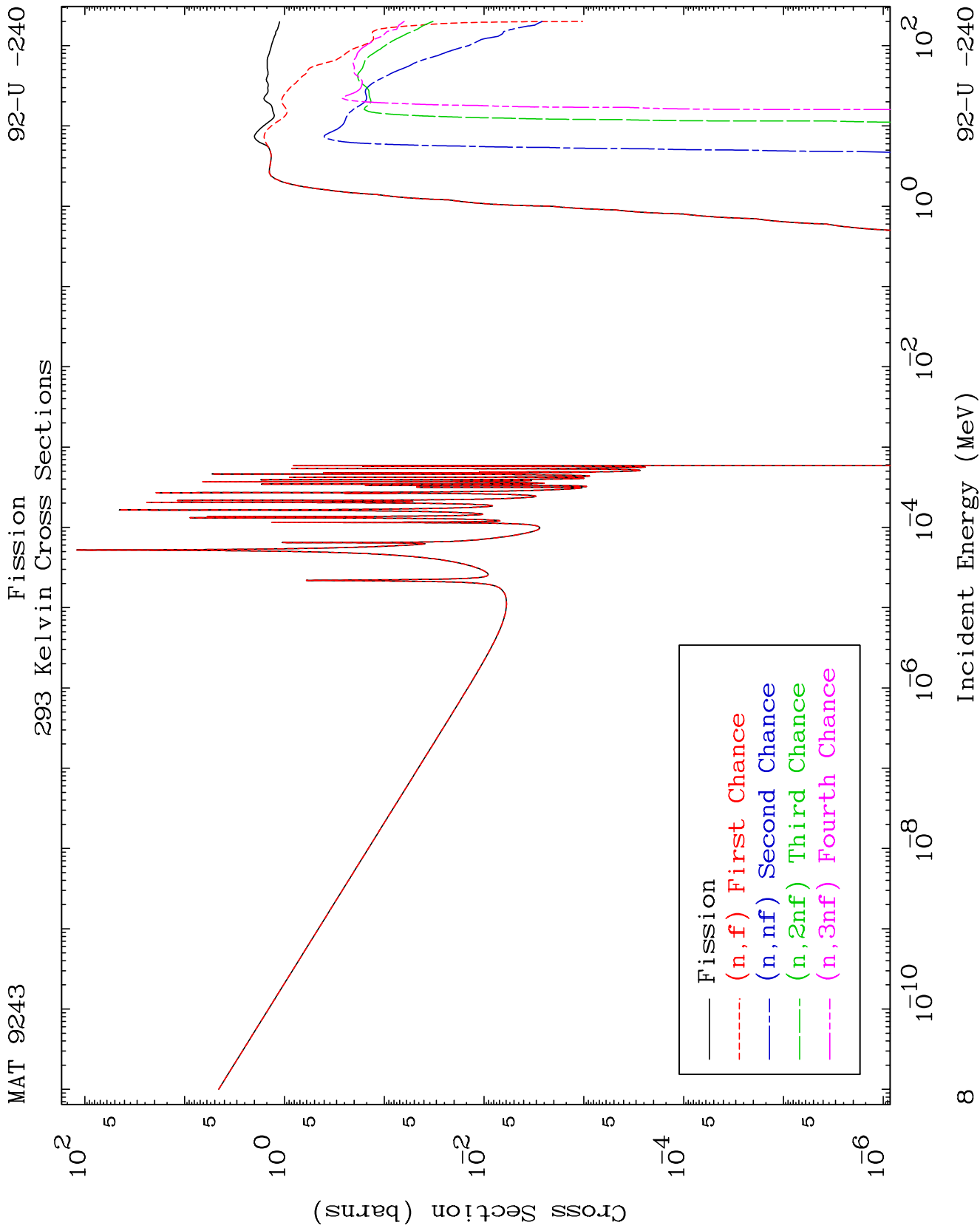


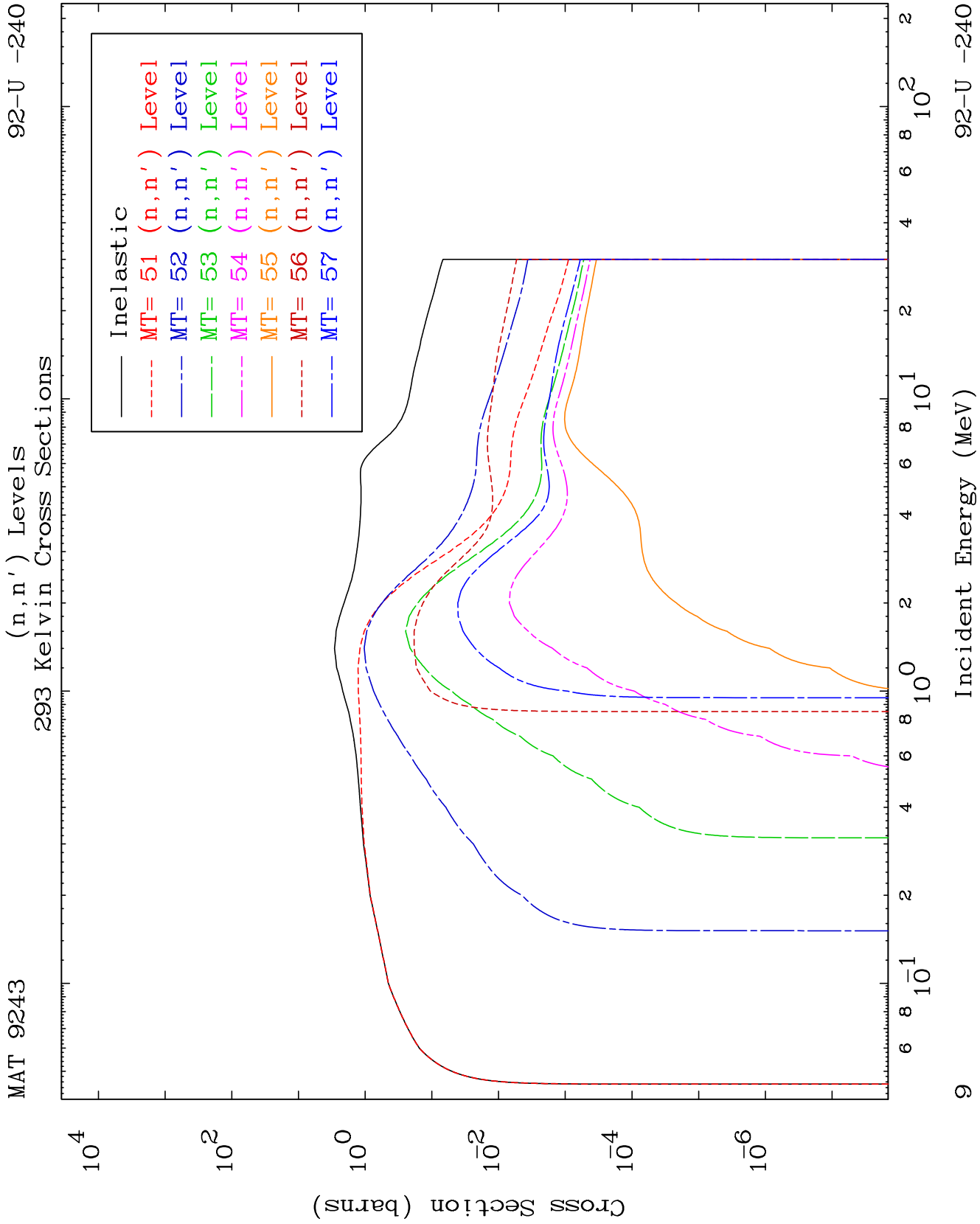


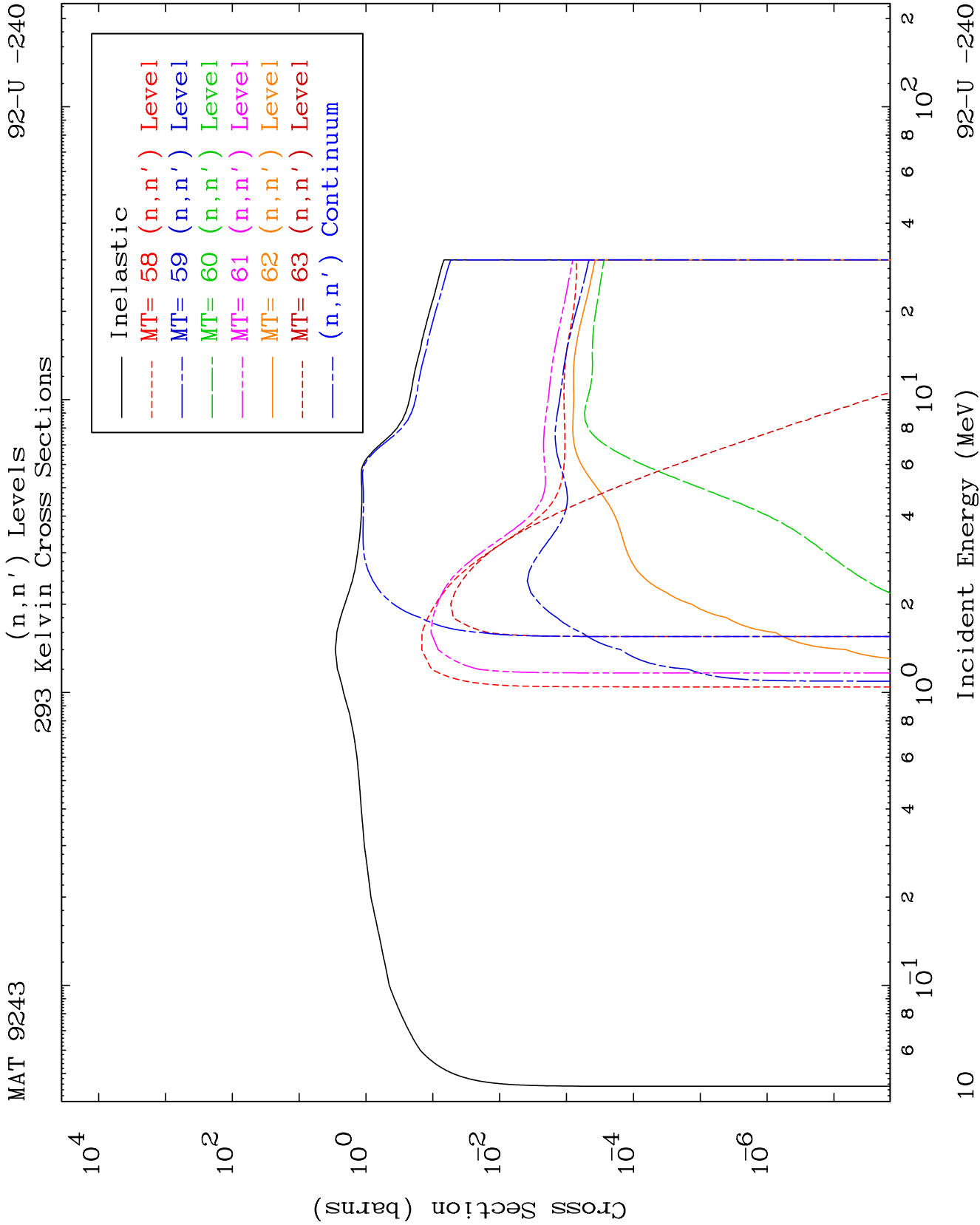


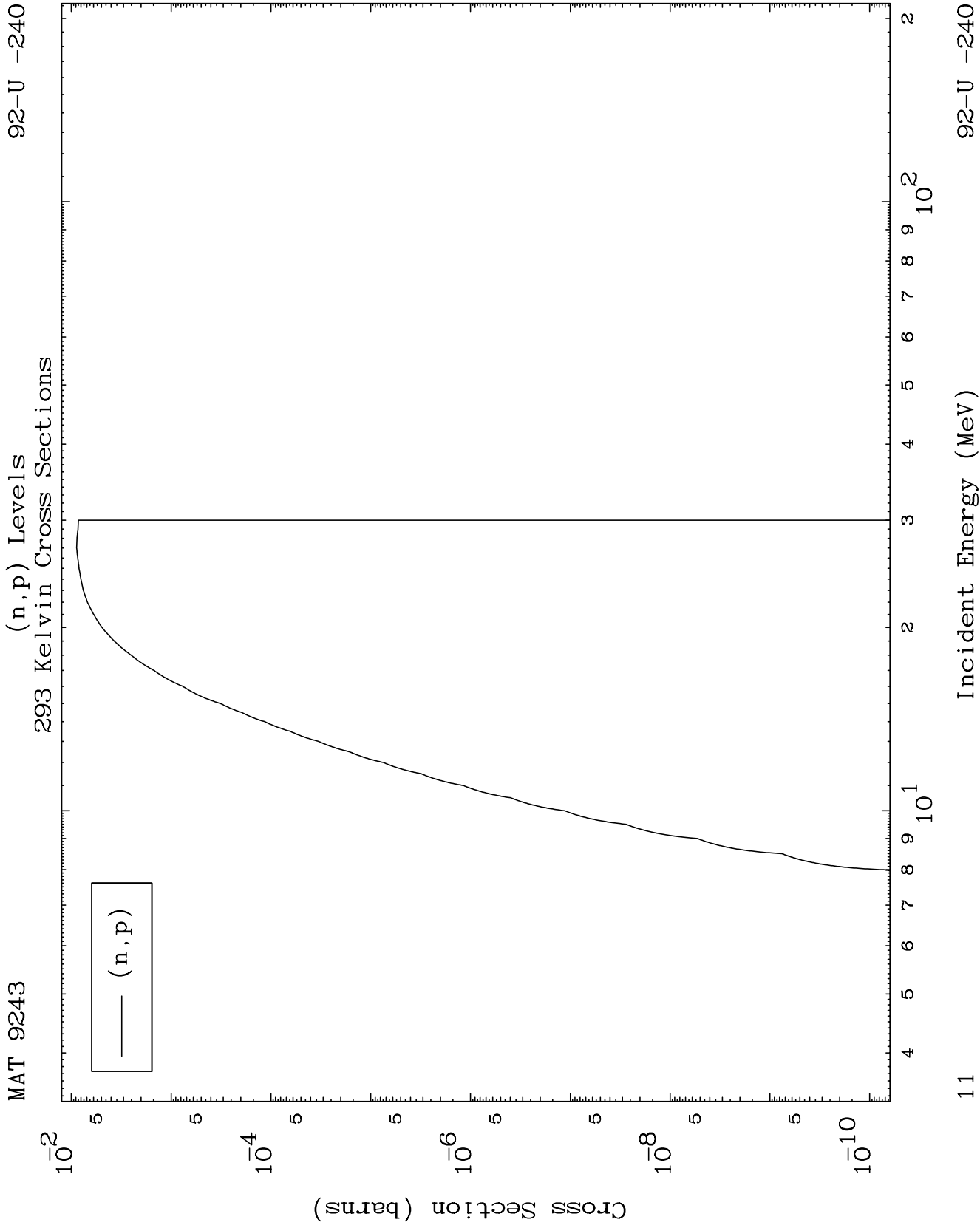








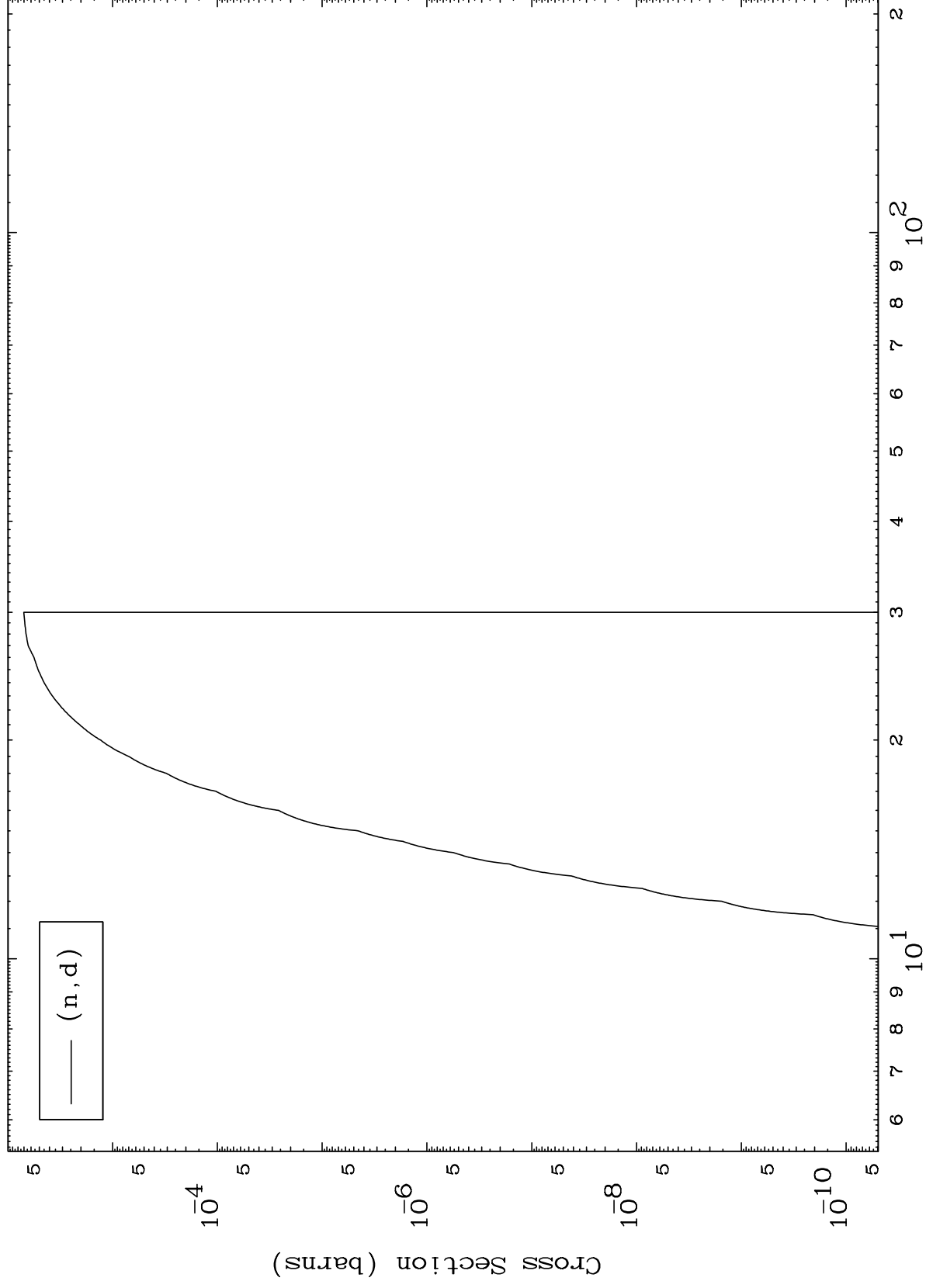




MAT 9243

(n,d) Levels
293 Kelvin Cross Sections

92-U -240



12

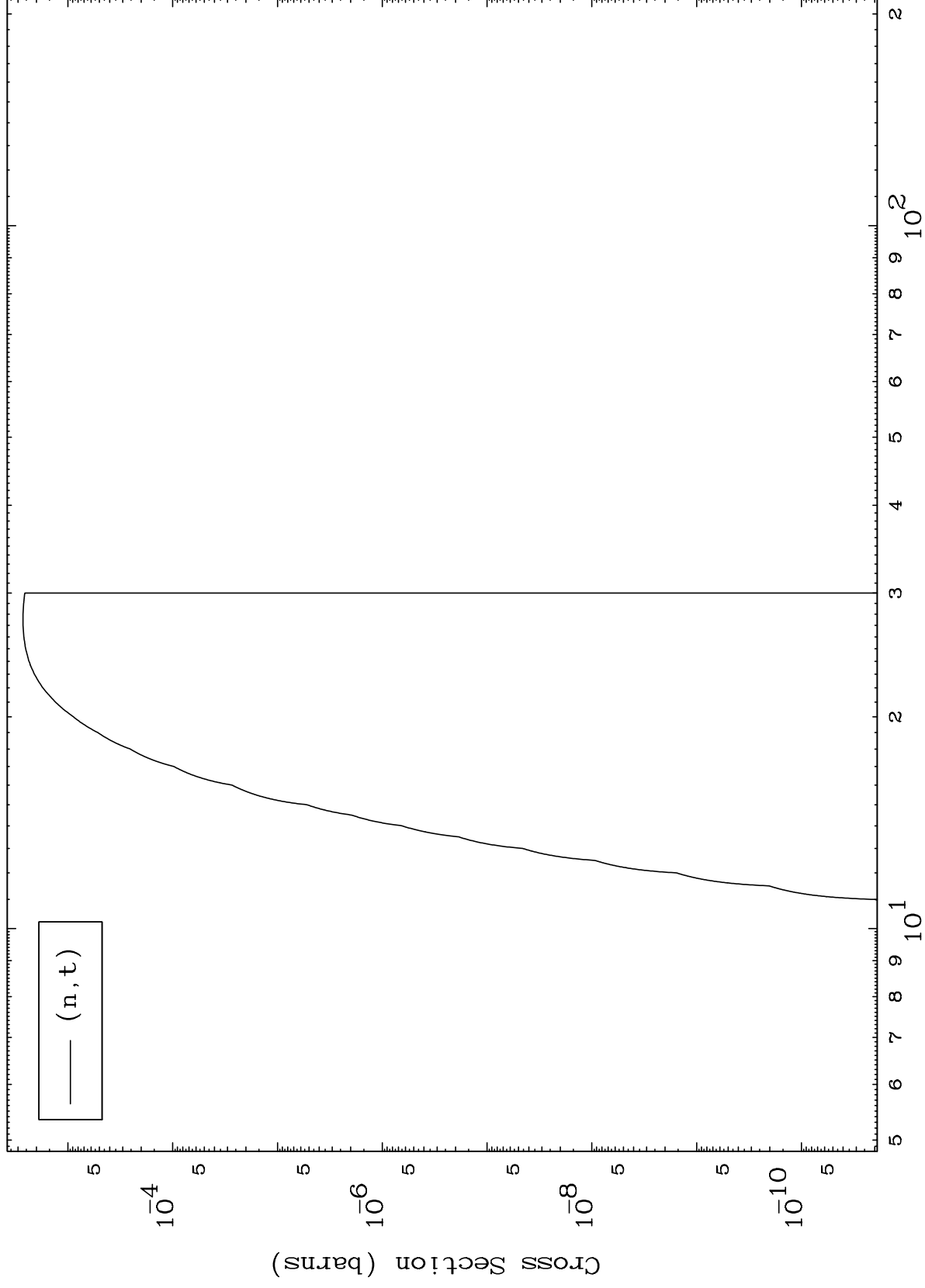
Incident Energy (MeV)

92-U -240

MAT 9243

(n,t) Levels
293 Kelvin Cross Sections

92-U -240



13

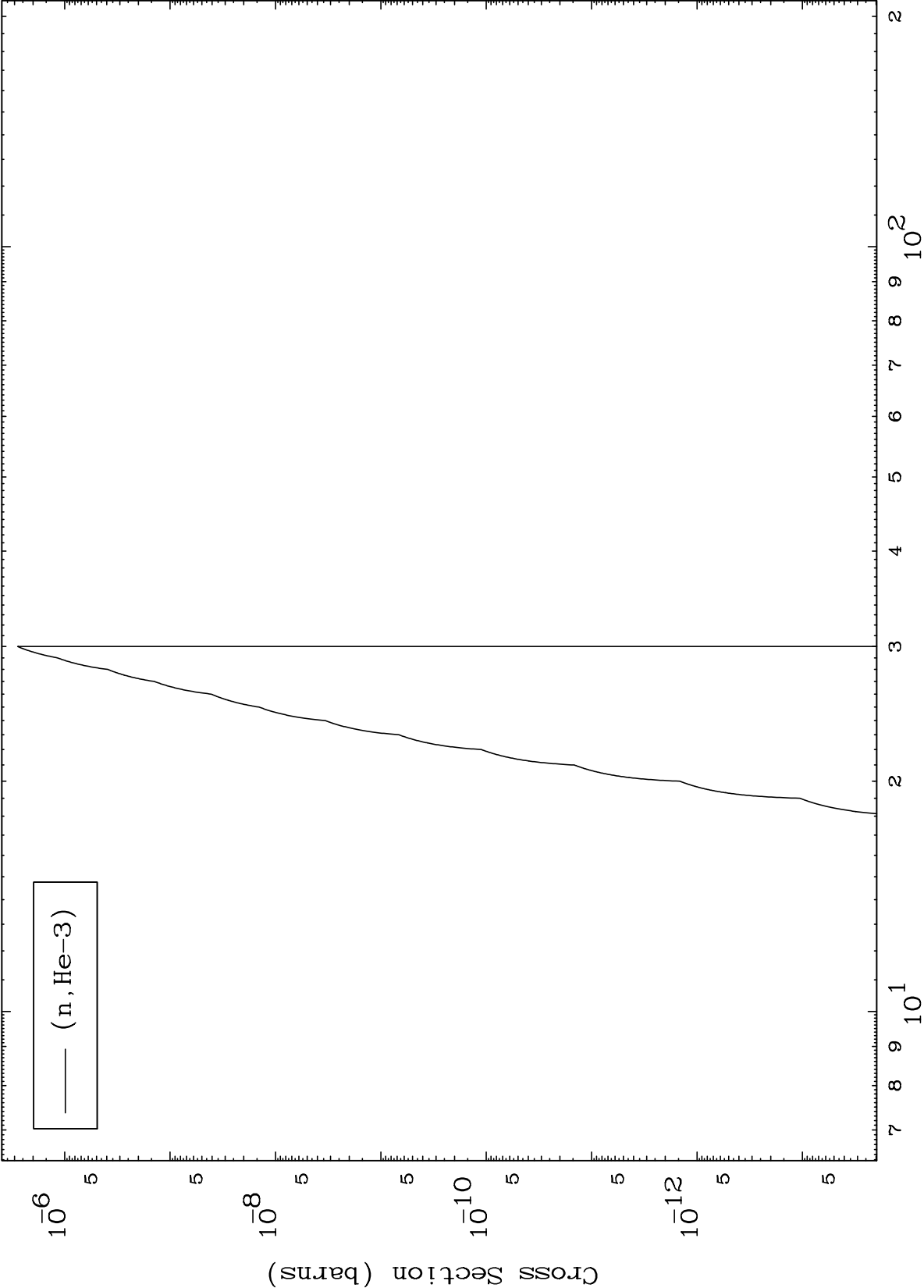
Incident Energy (MeV)

92-U -240

MAT 9243

(n,He3) Levels
293 Kelvin Cross Sections

92-U -240



14

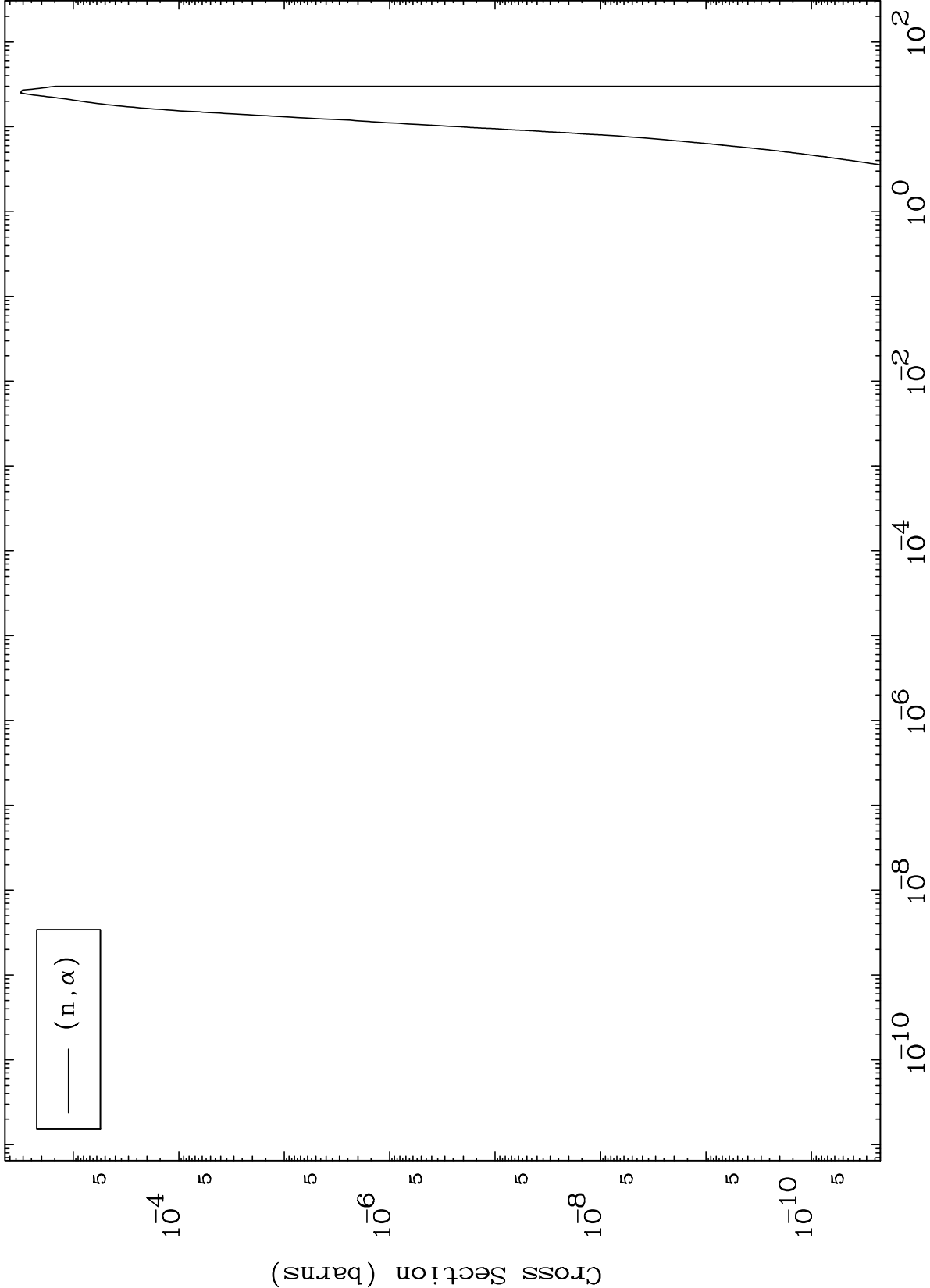
Incident Energy (MeV)

92-U -240

MAT 9243

(n, α) Levels
293 Kelvin Cross Sections

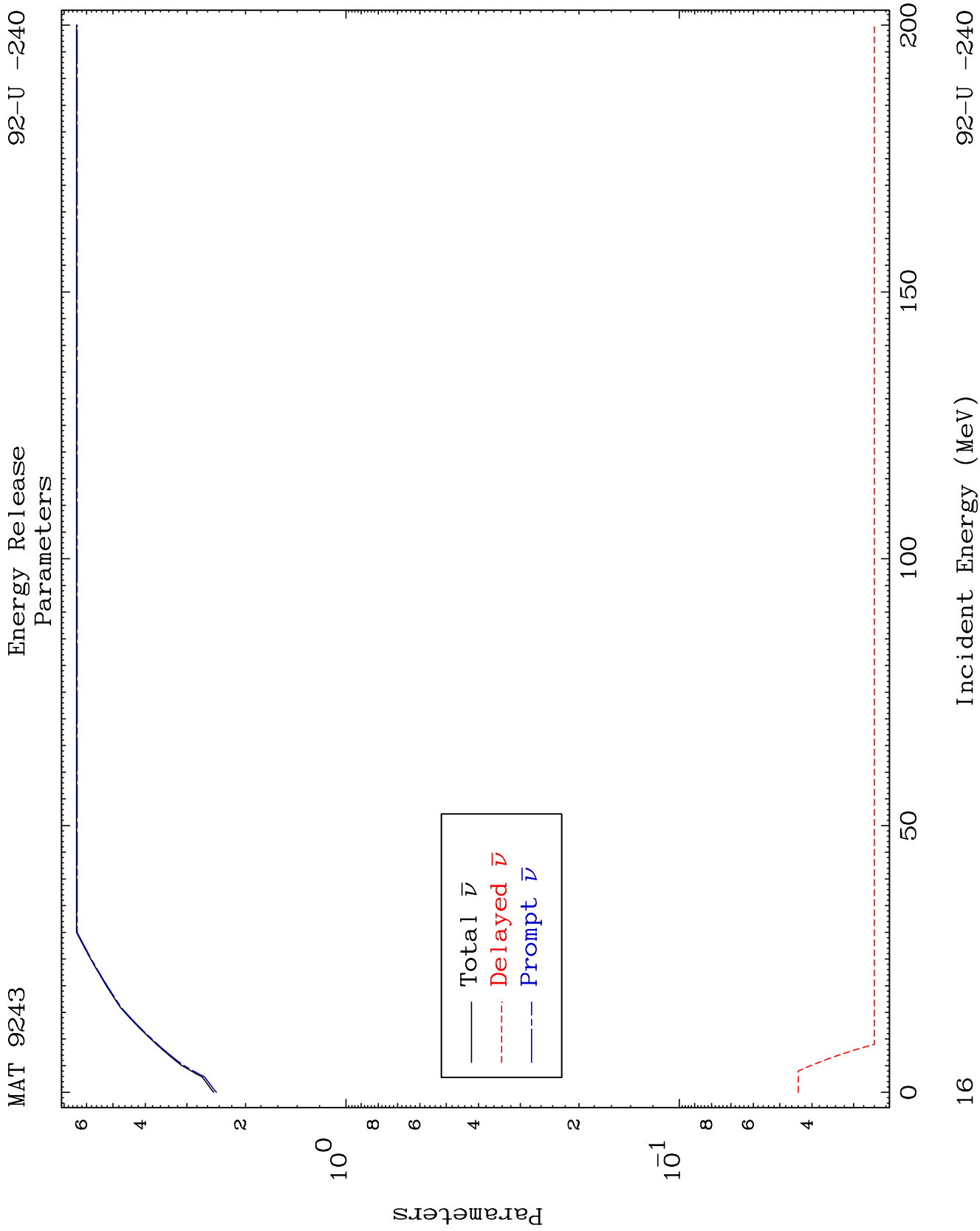
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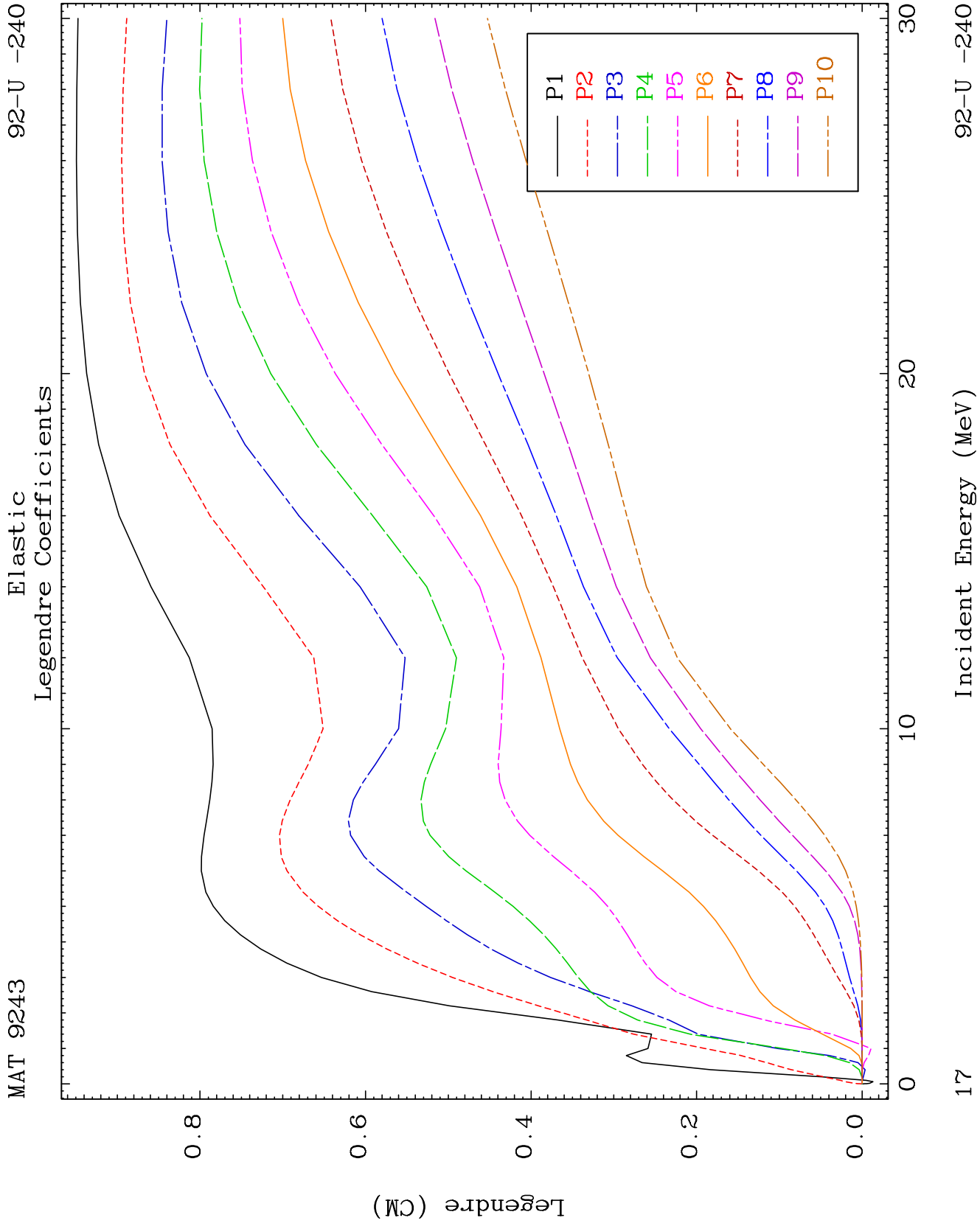


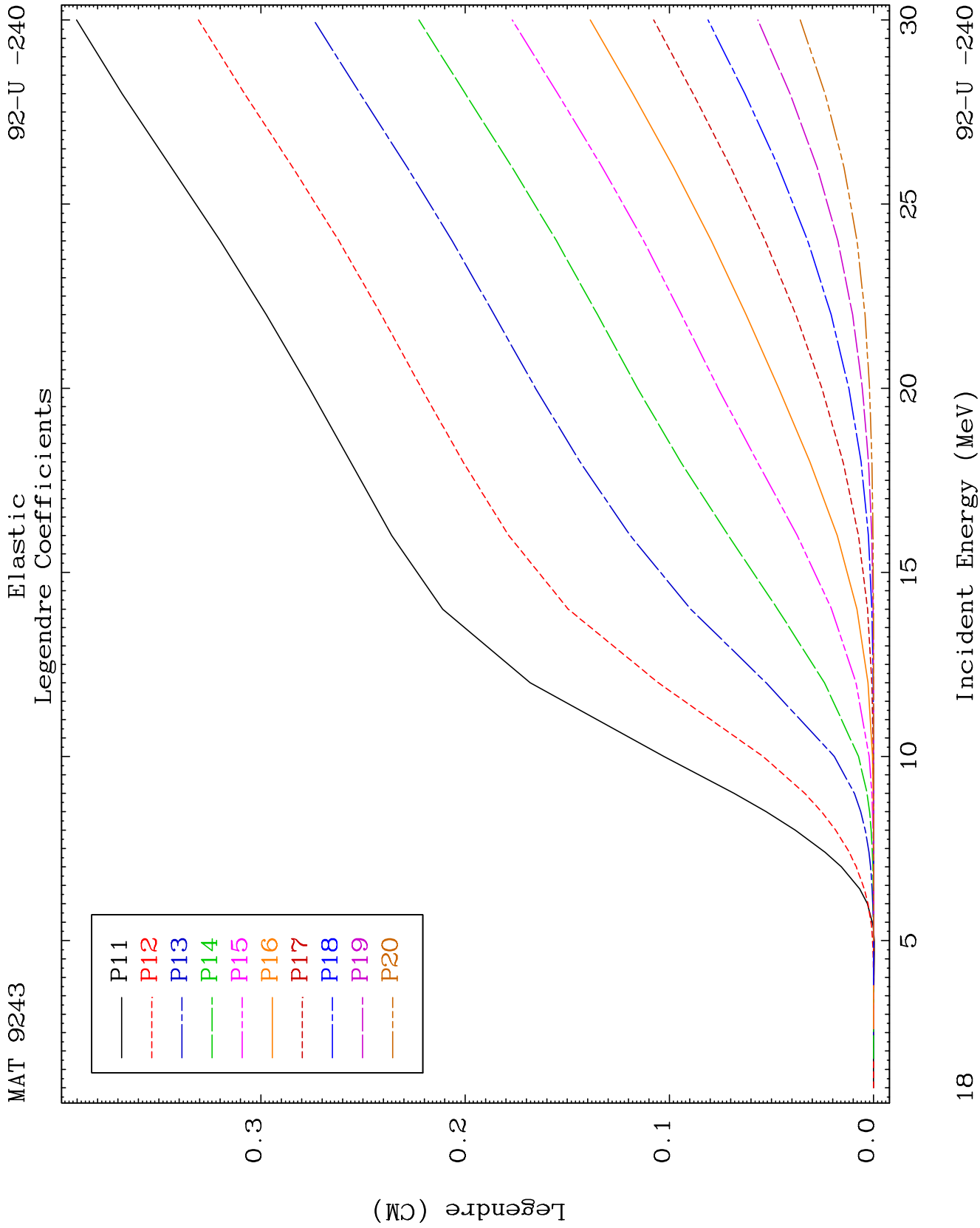
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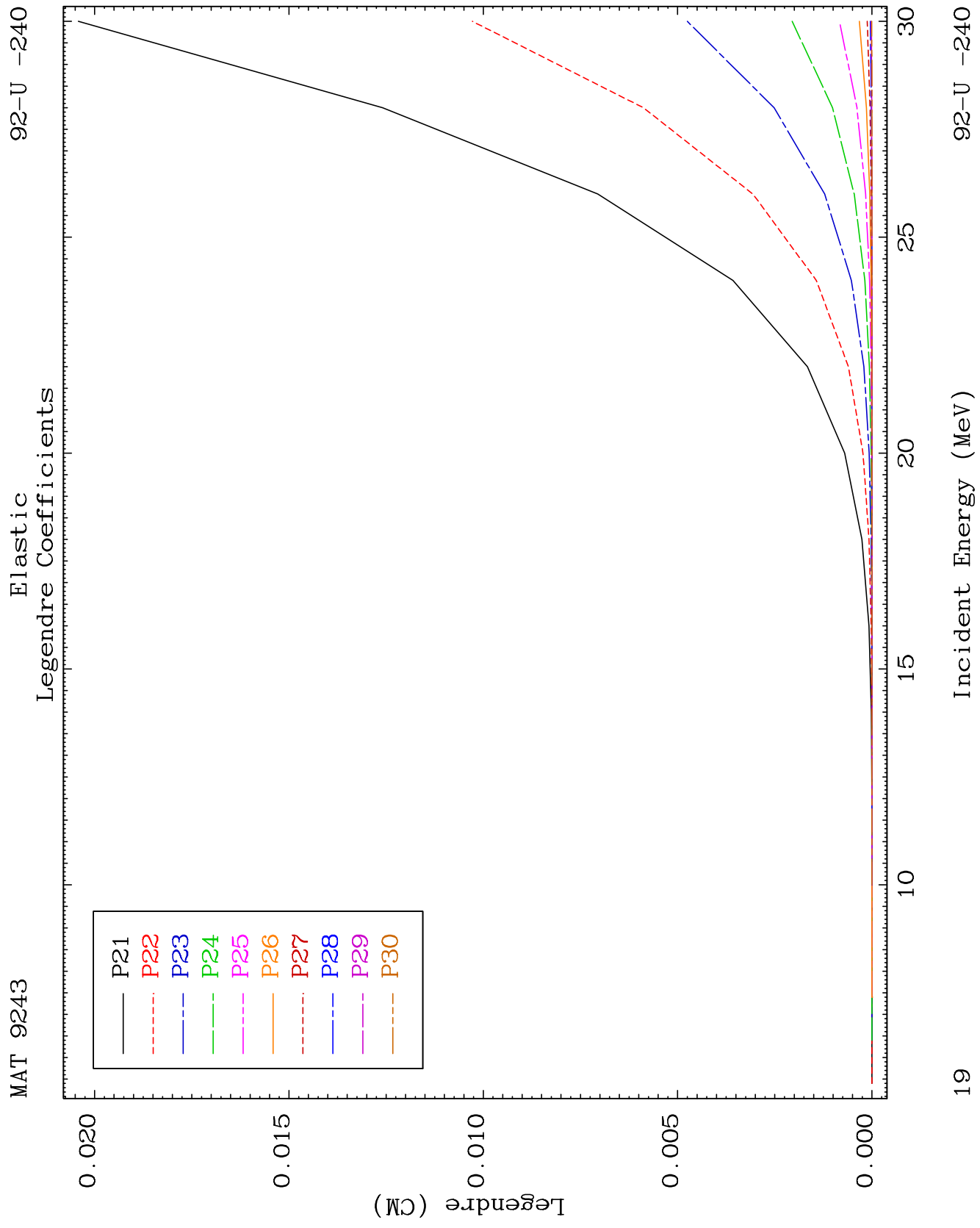
Incident Energy (MeV)

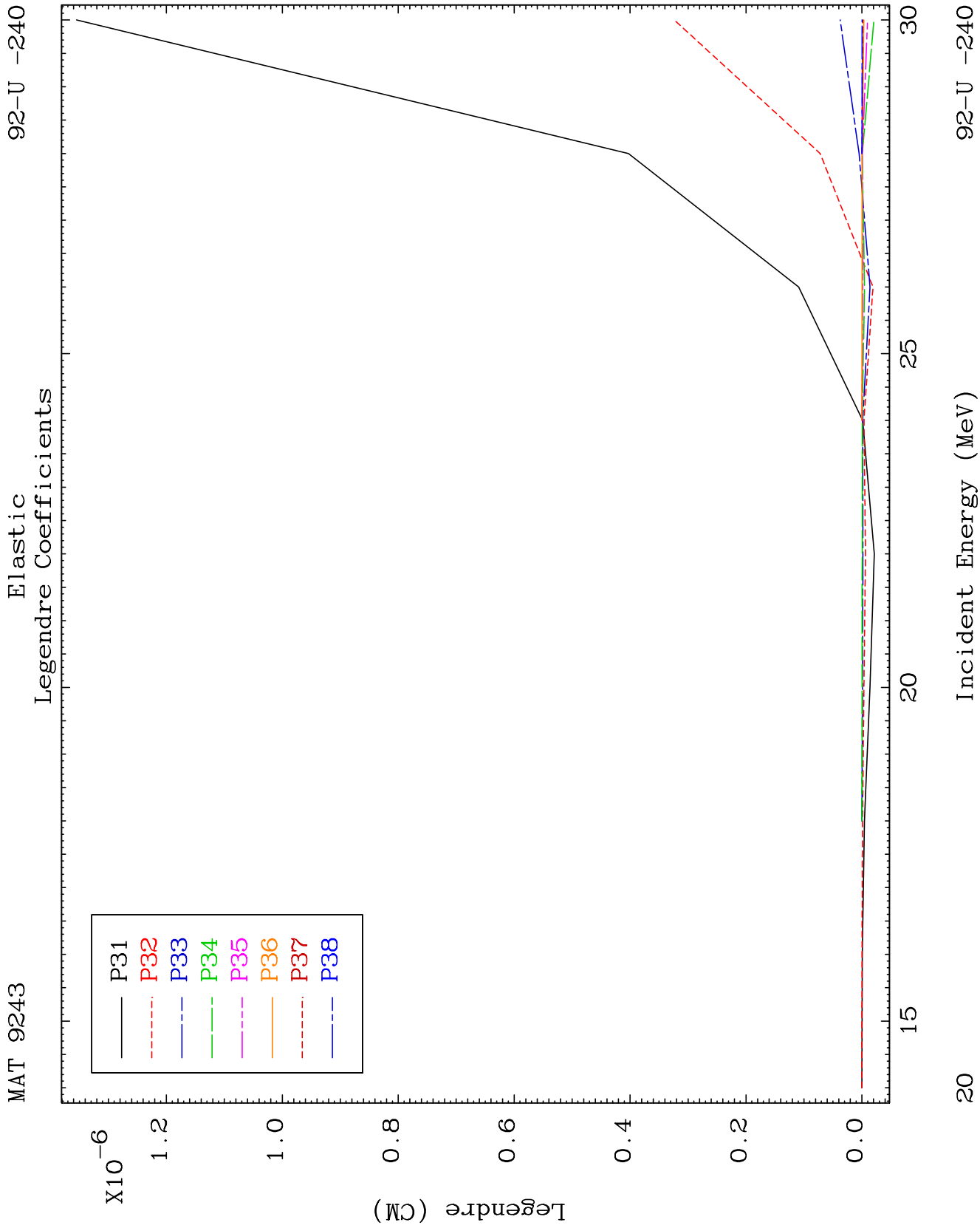
92-U -240



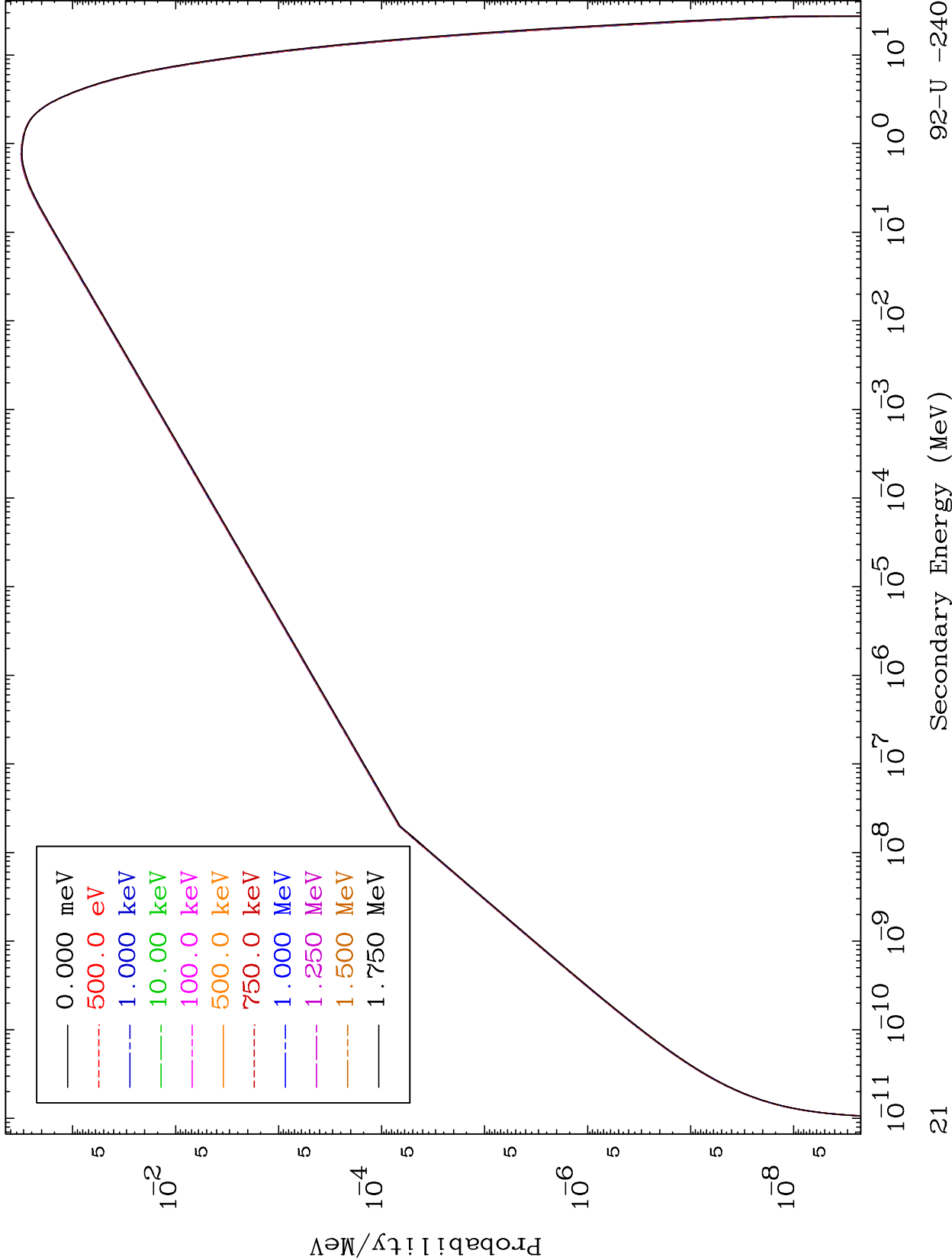








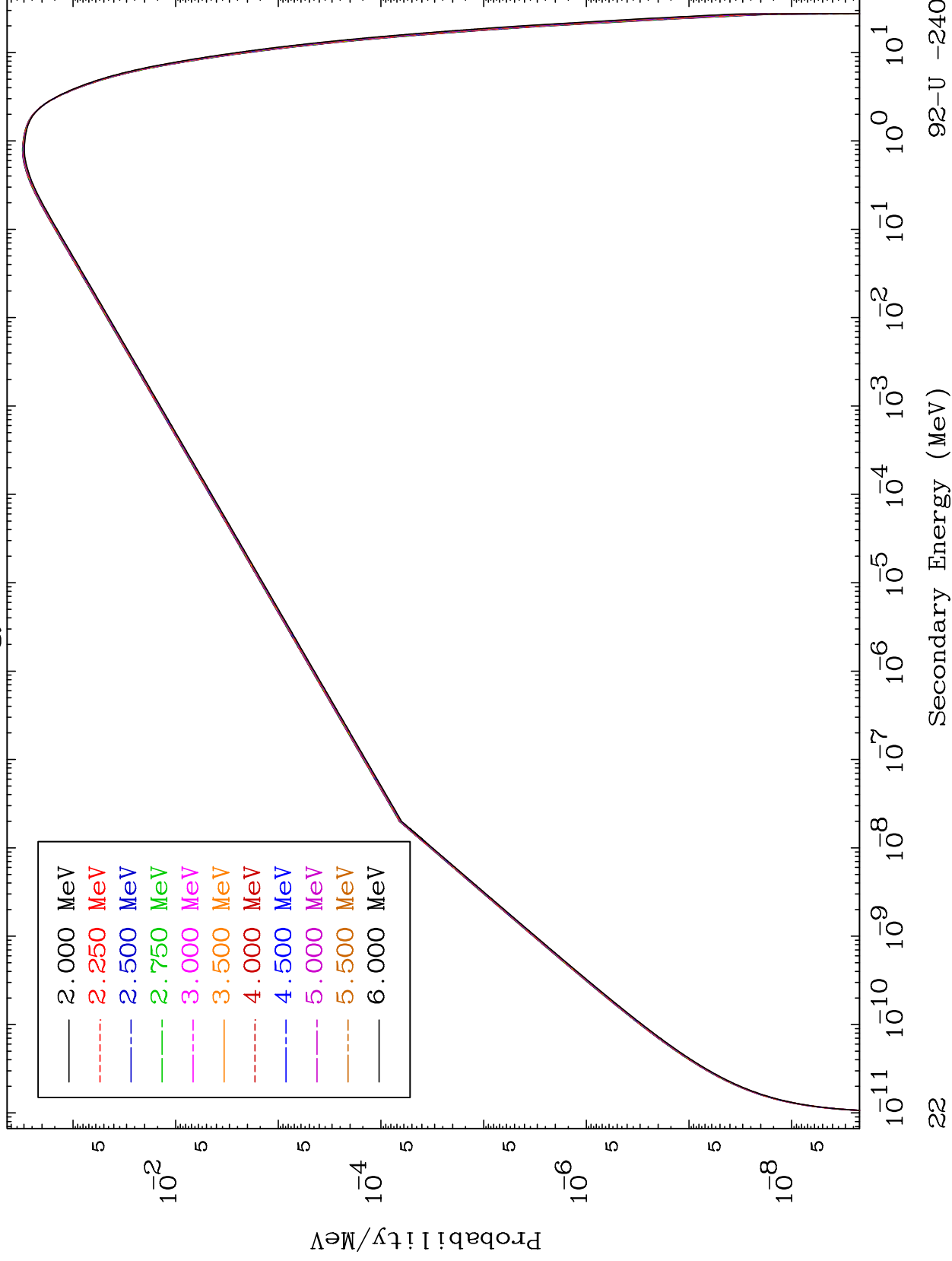
Fission
Energy Distributions



MAT 9243

Fission
Energy Distributions

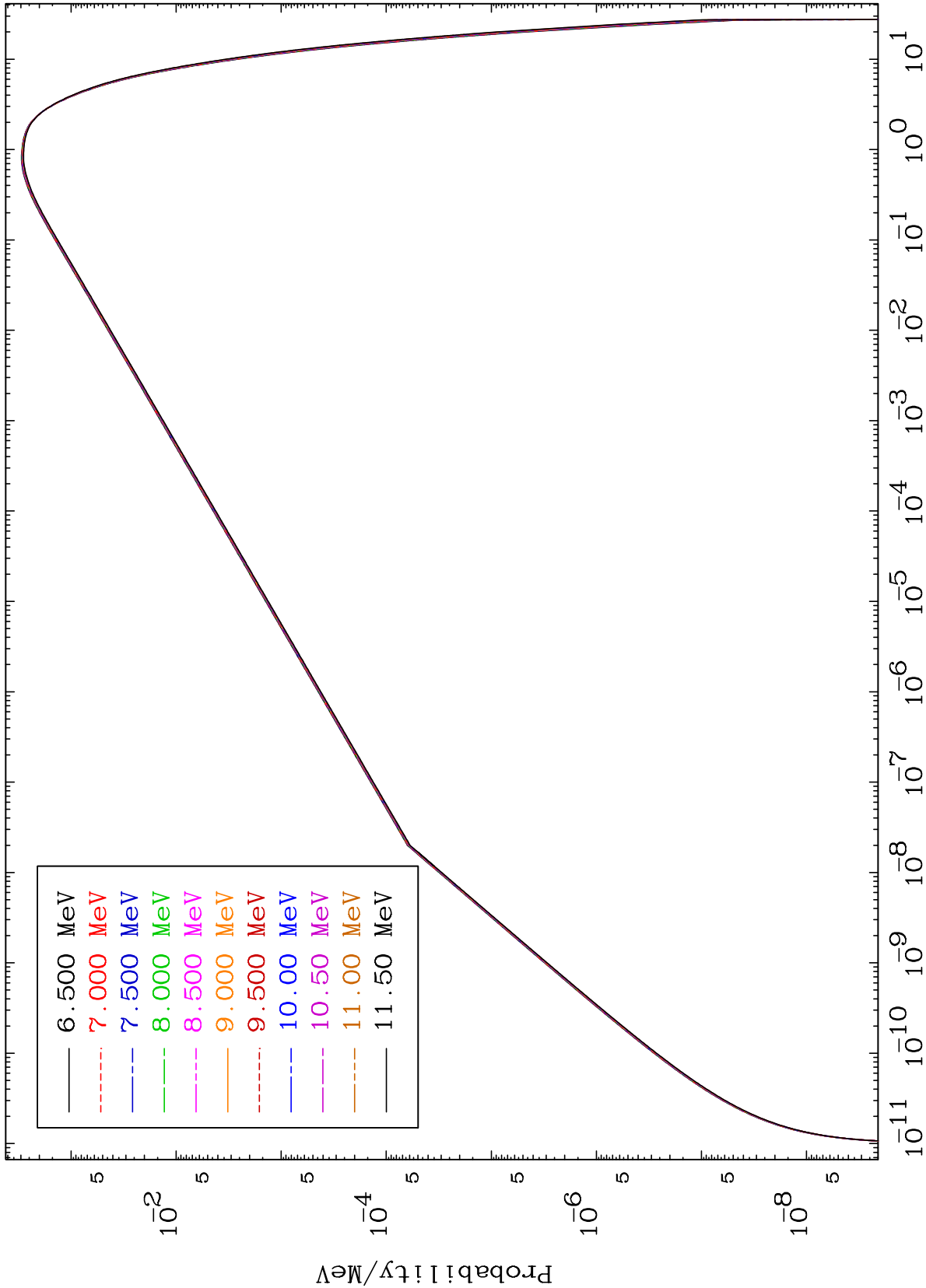
92-U -240



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Fission
Energy Distributions

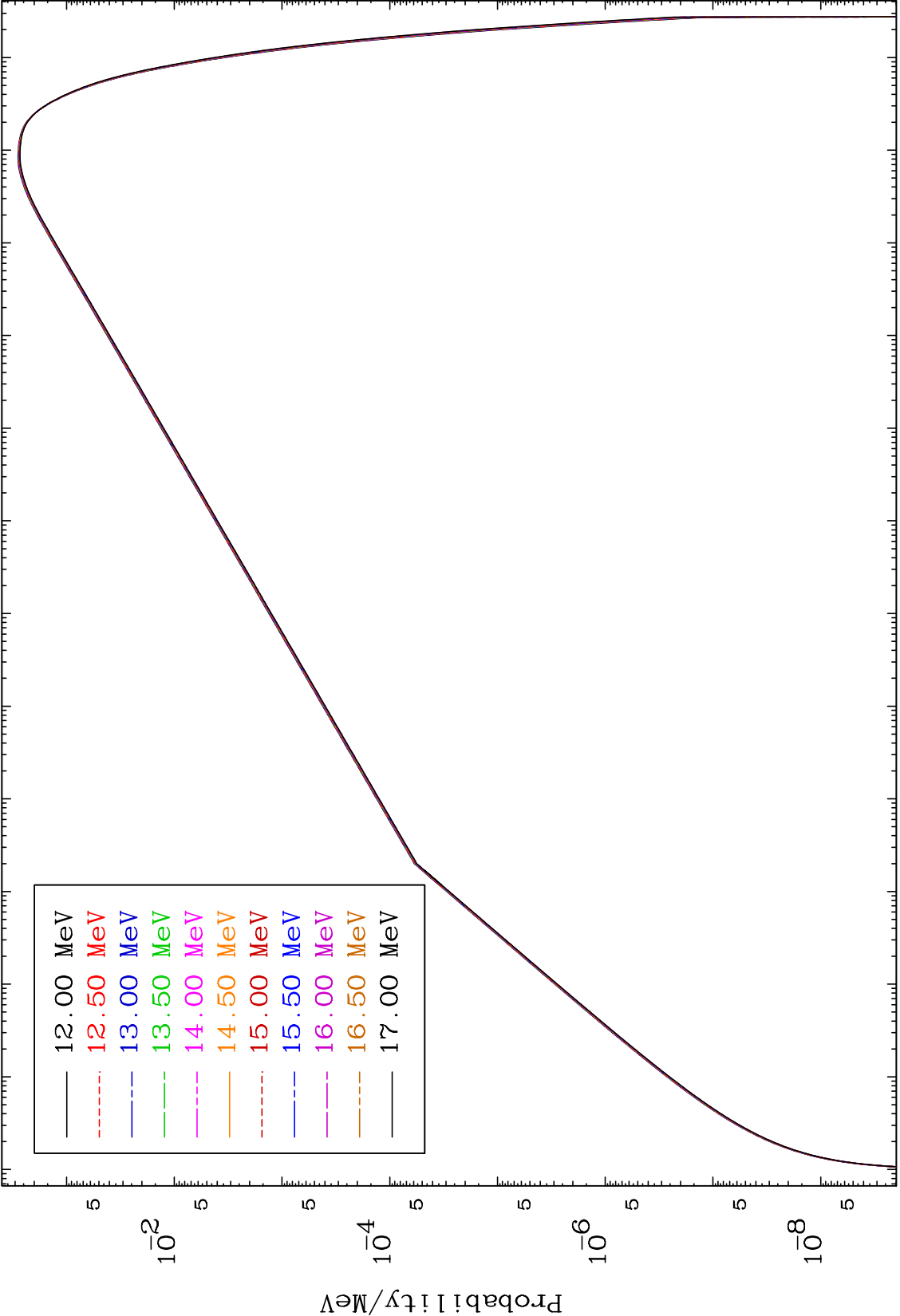
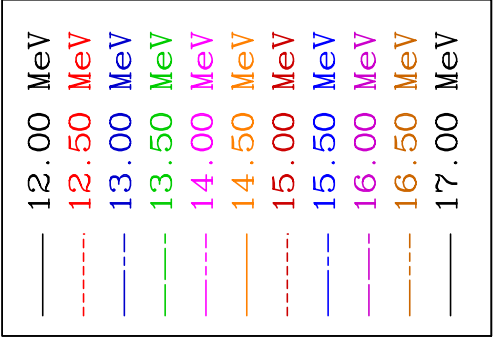
92-U -240



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Fission
Energy Distributions

92-U -240



Secondary Energy (MeV)

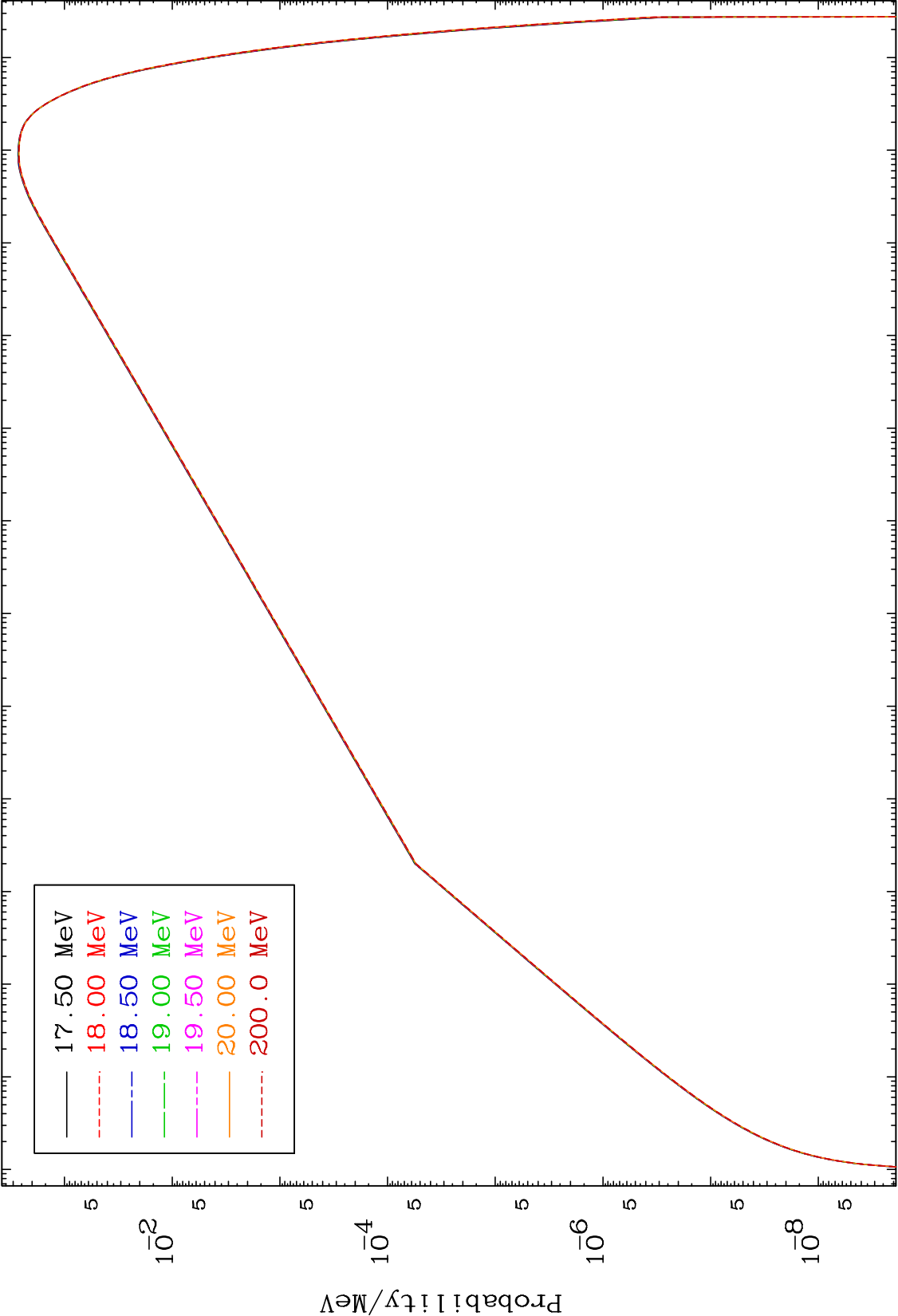
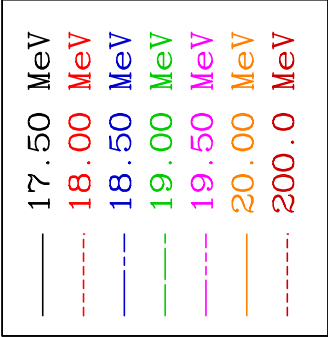
92-U -240

24

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Fission
Energy Distributions

92-U -240



Secondary Energy (MeV)

92-U -240

25