

Program EVALPLOT
(Version 2021-1)

by

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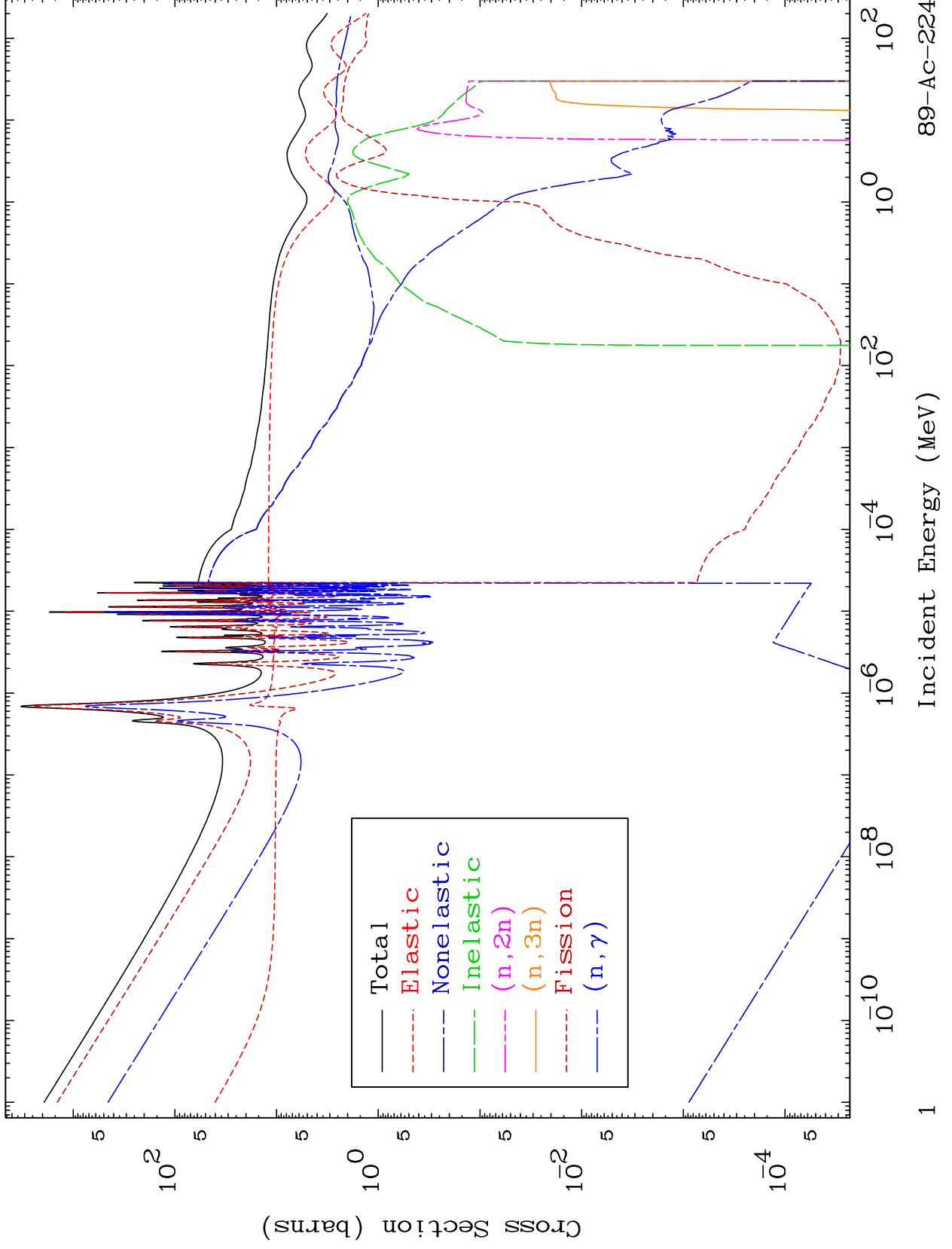
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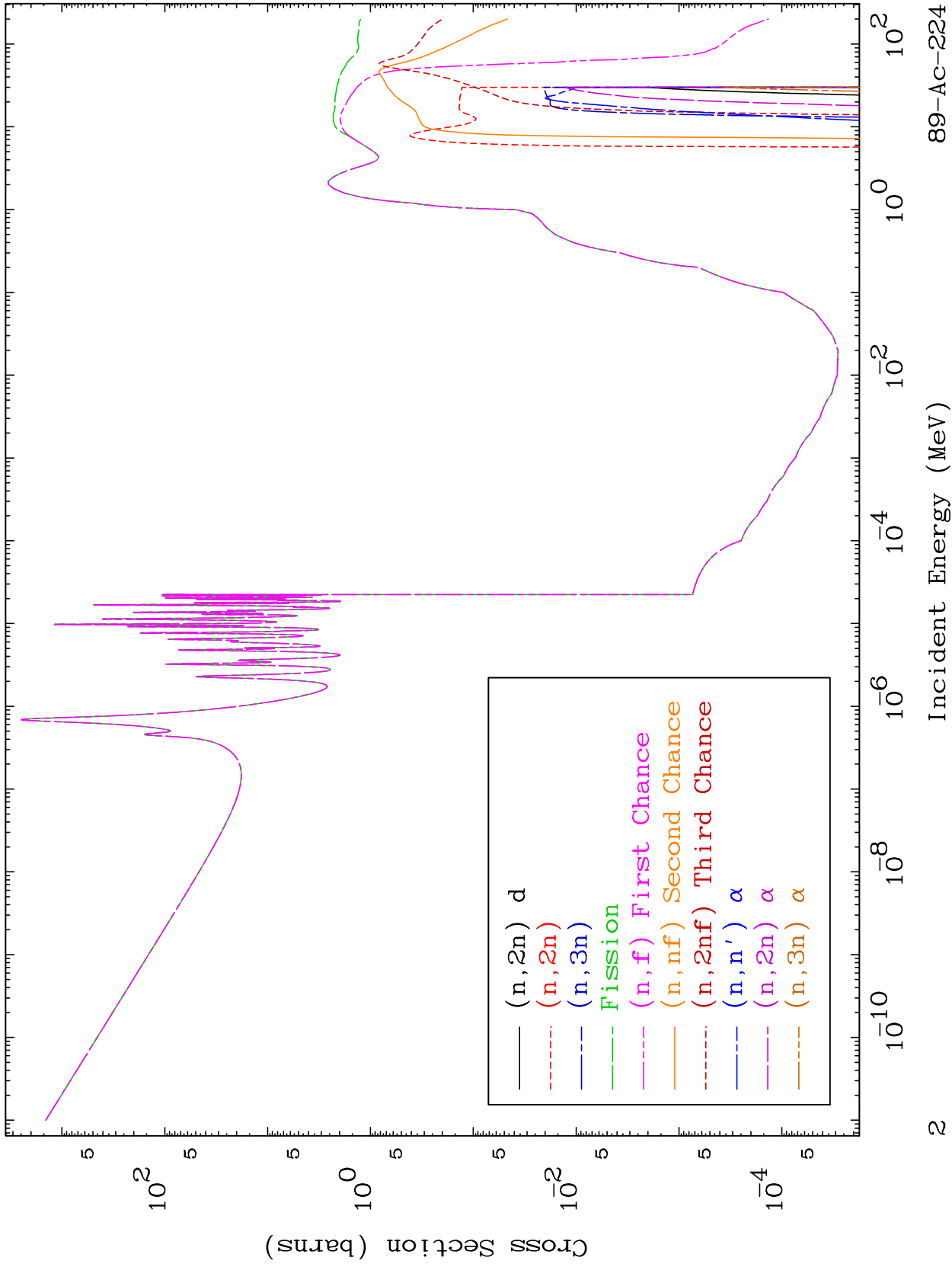
Press Mouse Button to Start

MAT 8922

Neutron Major
293 Kelvin Cross Sections

89-Ac-224

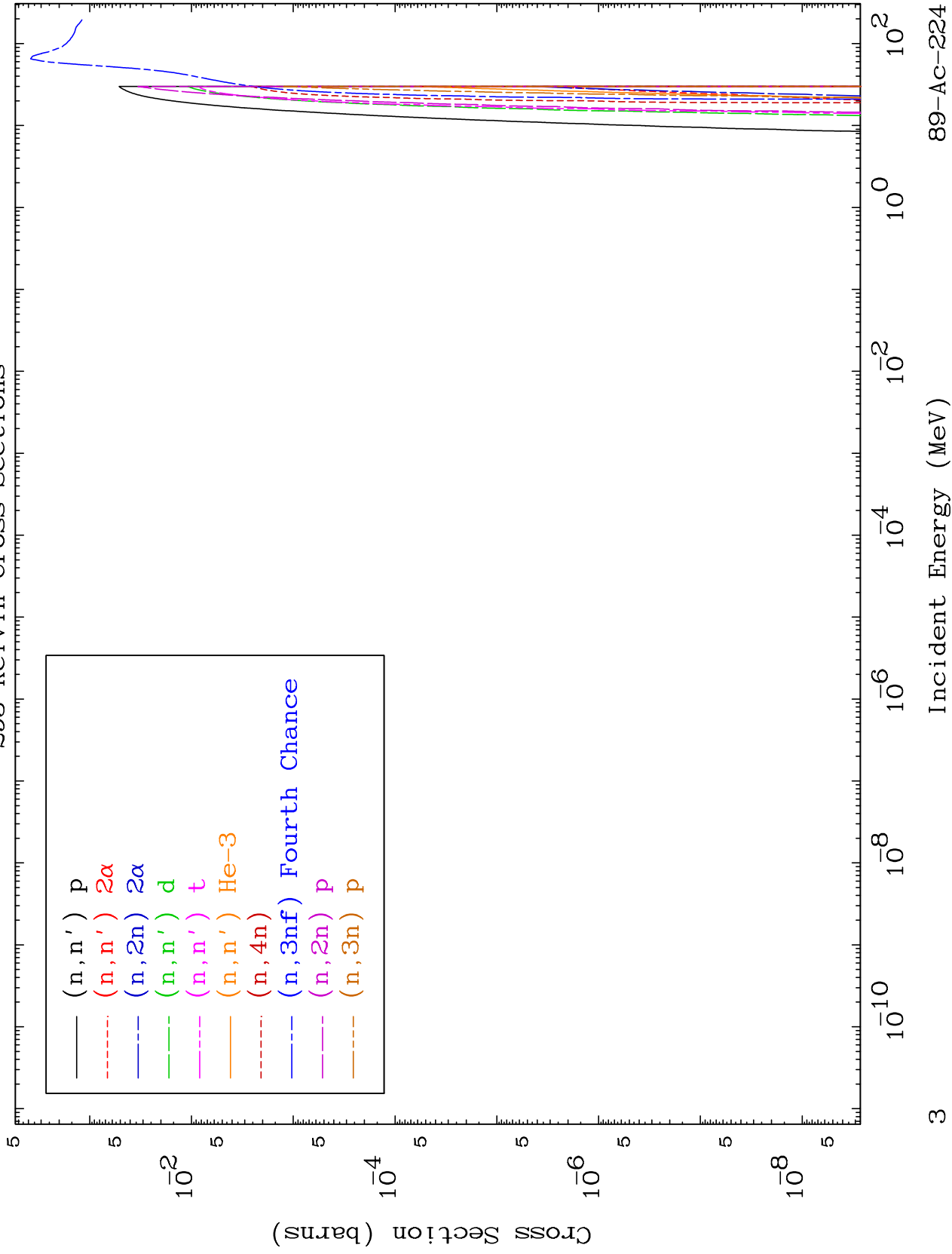




MAT 8922

Neutron Absorption
293 Kelvin Cross Sections

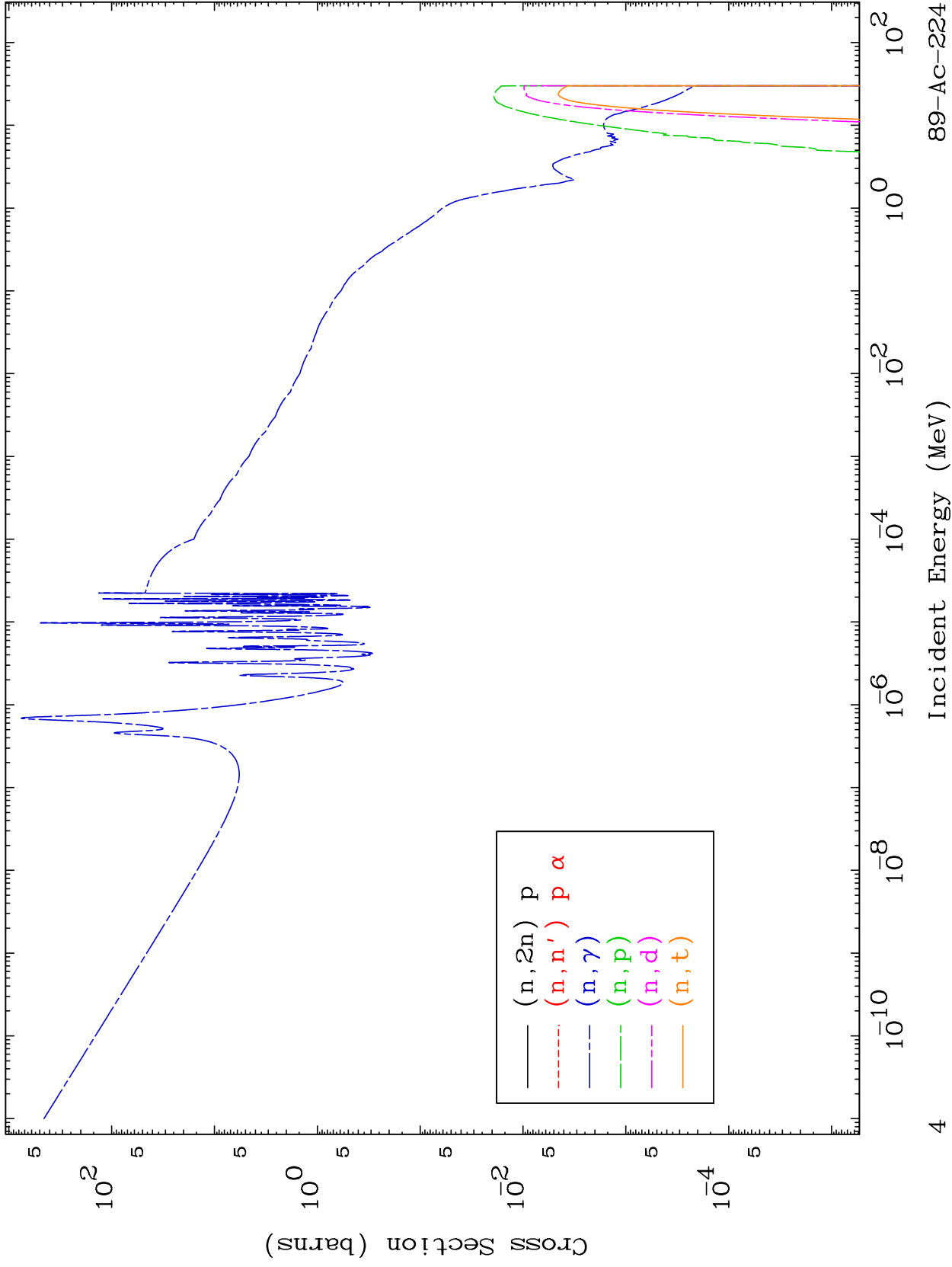
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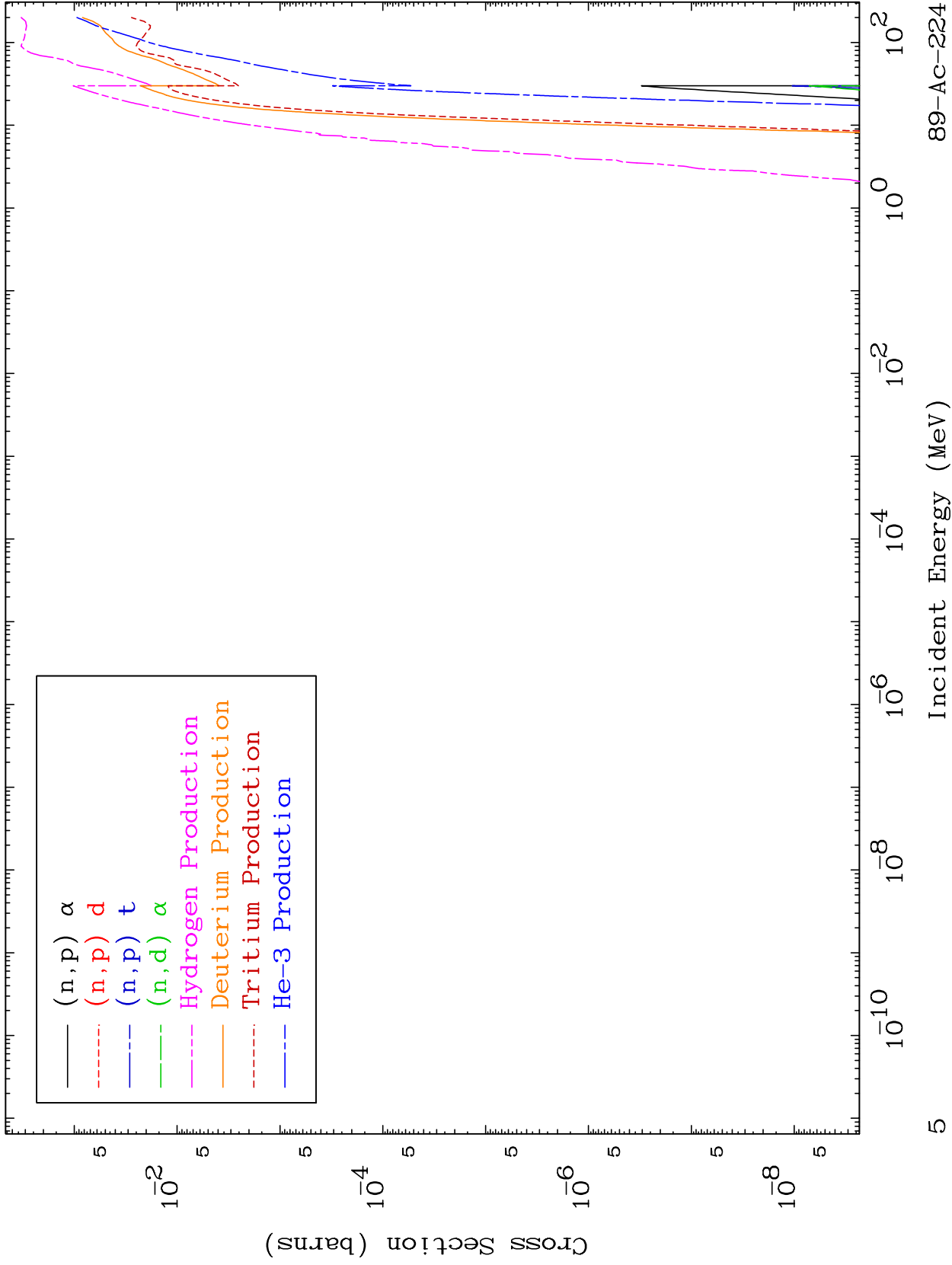


MAT 89222

Neutron Absorption
293 Kelvin Cross Sections

89-Ac-224

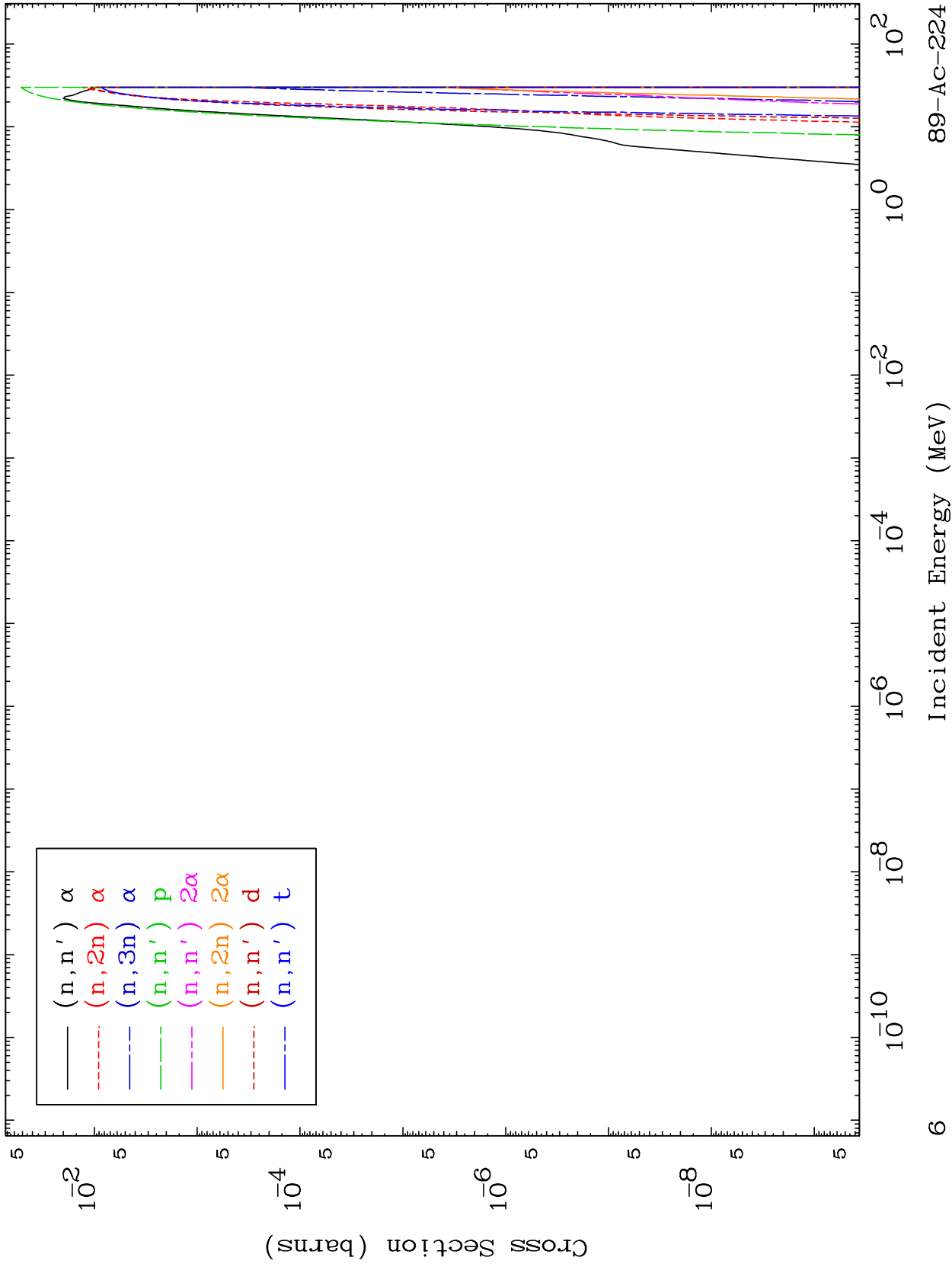




MAT 89222

Charged Particle
293 Kelvin Cross Sections

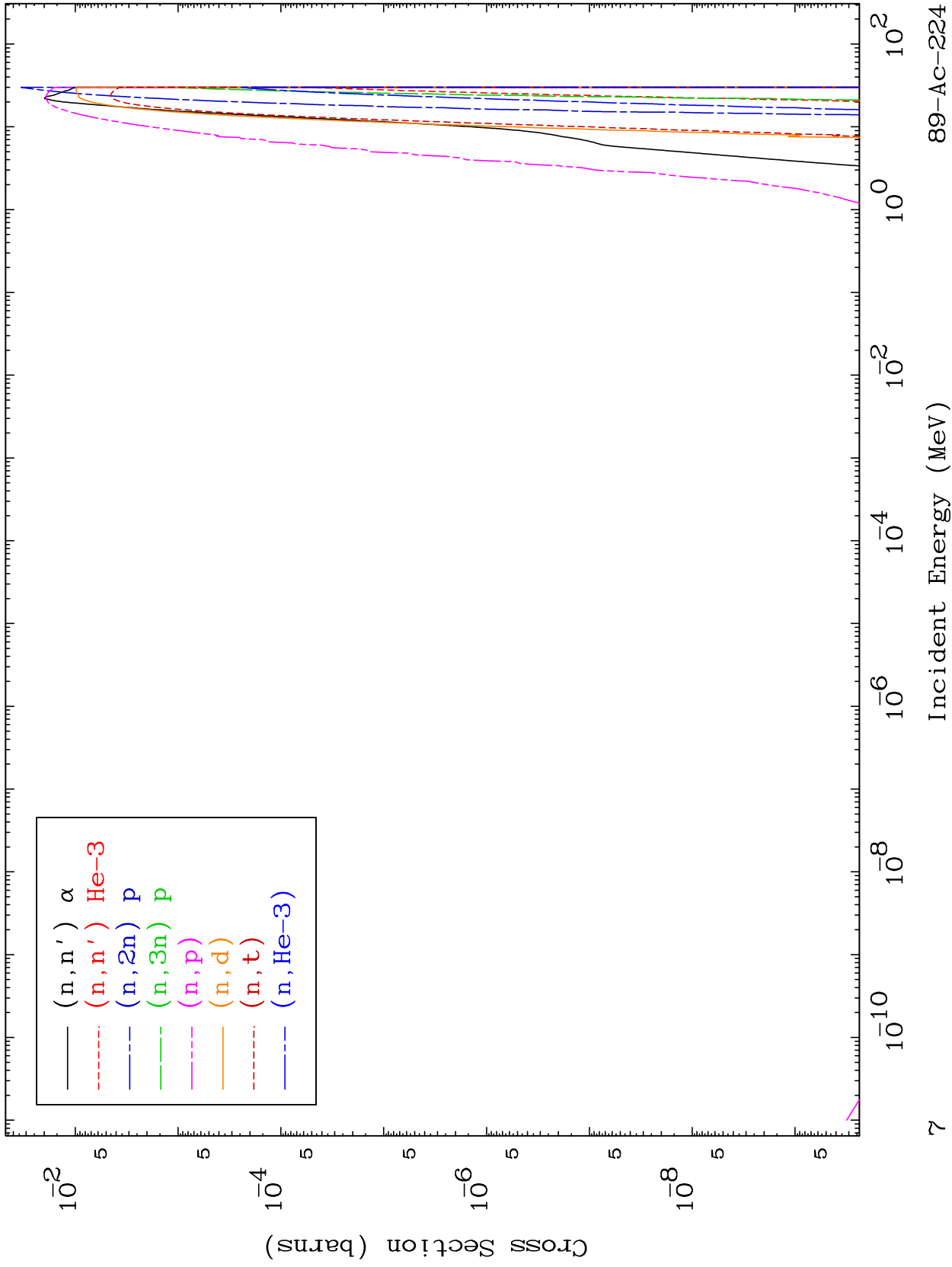
89-Ac-224



MAT 8922

Charged Particle
293 Kelvin Cross Sections

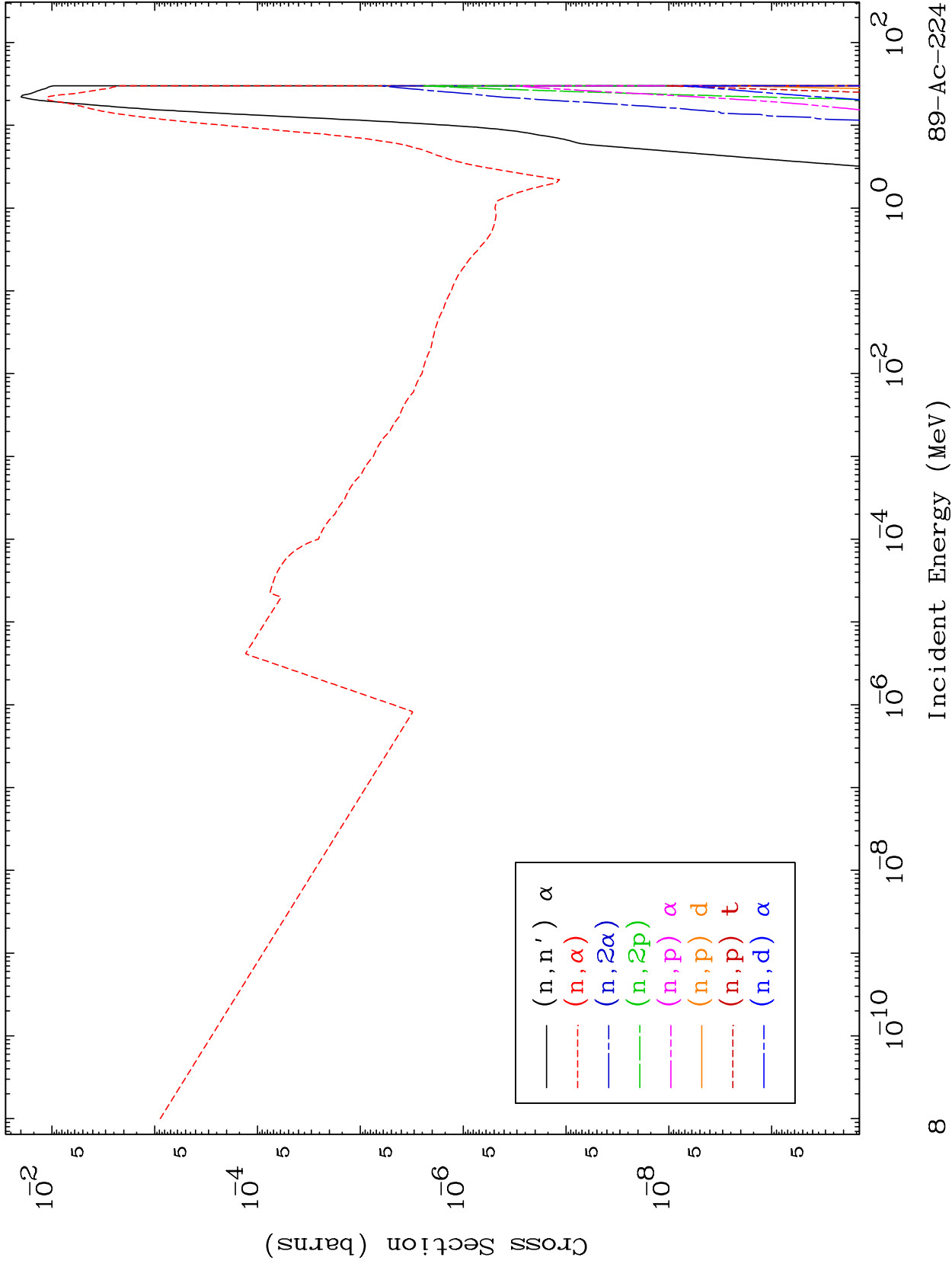
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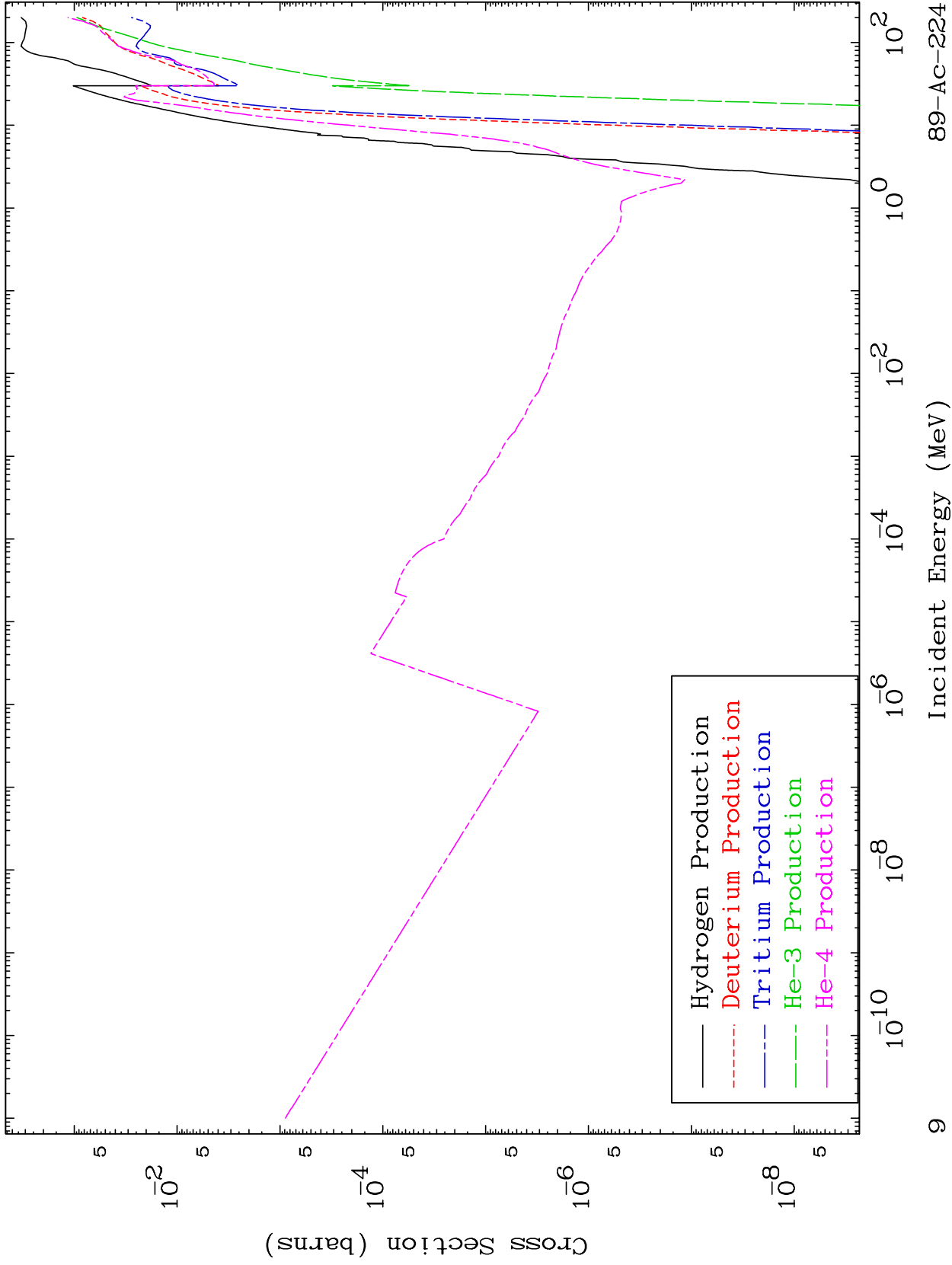


MAT 89222

Charged Particle
293 Kelvin Cross Sections

89-Ac-224

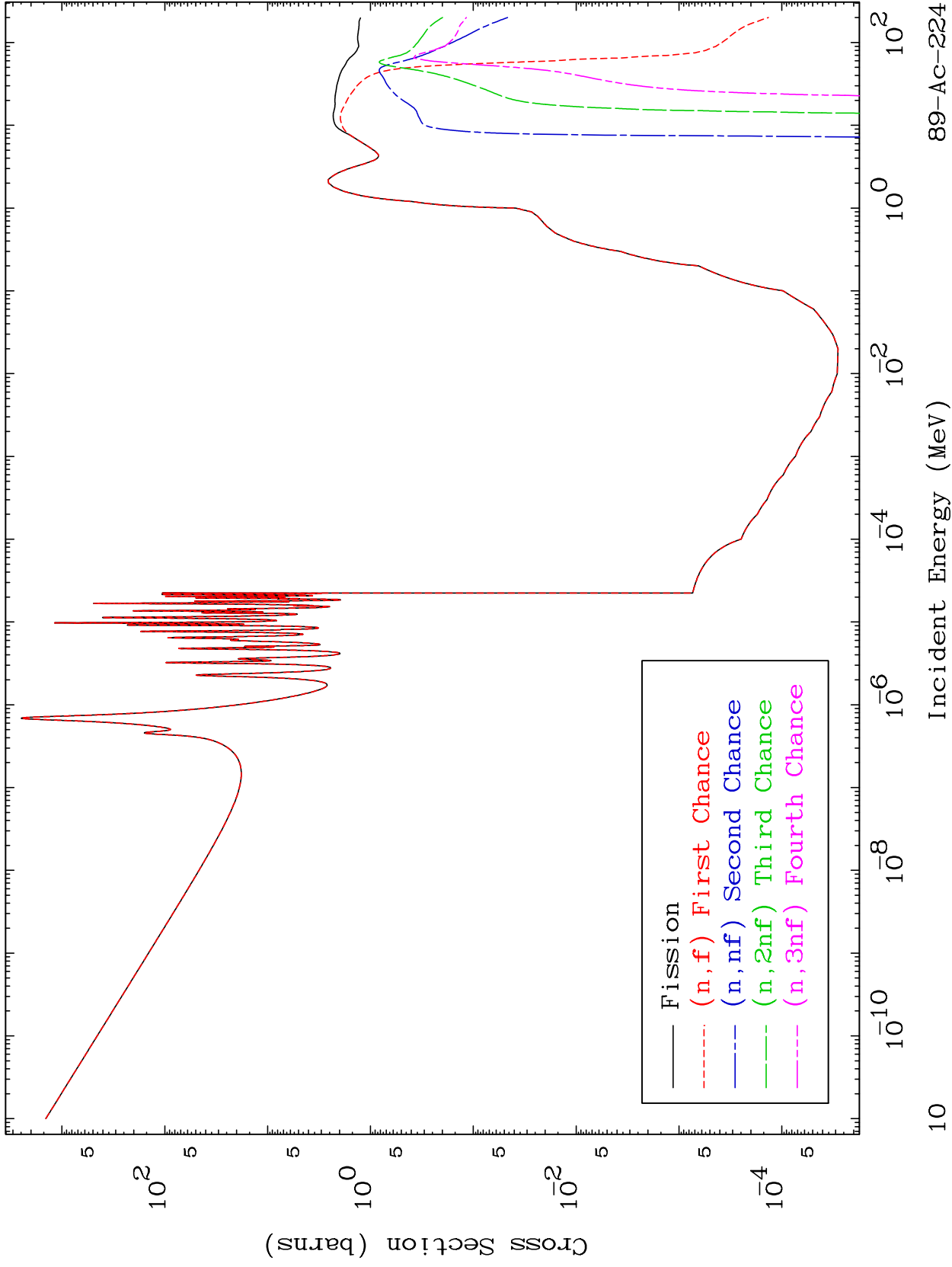




MAT 8922

Fission
293 Kelvin Cross Sections

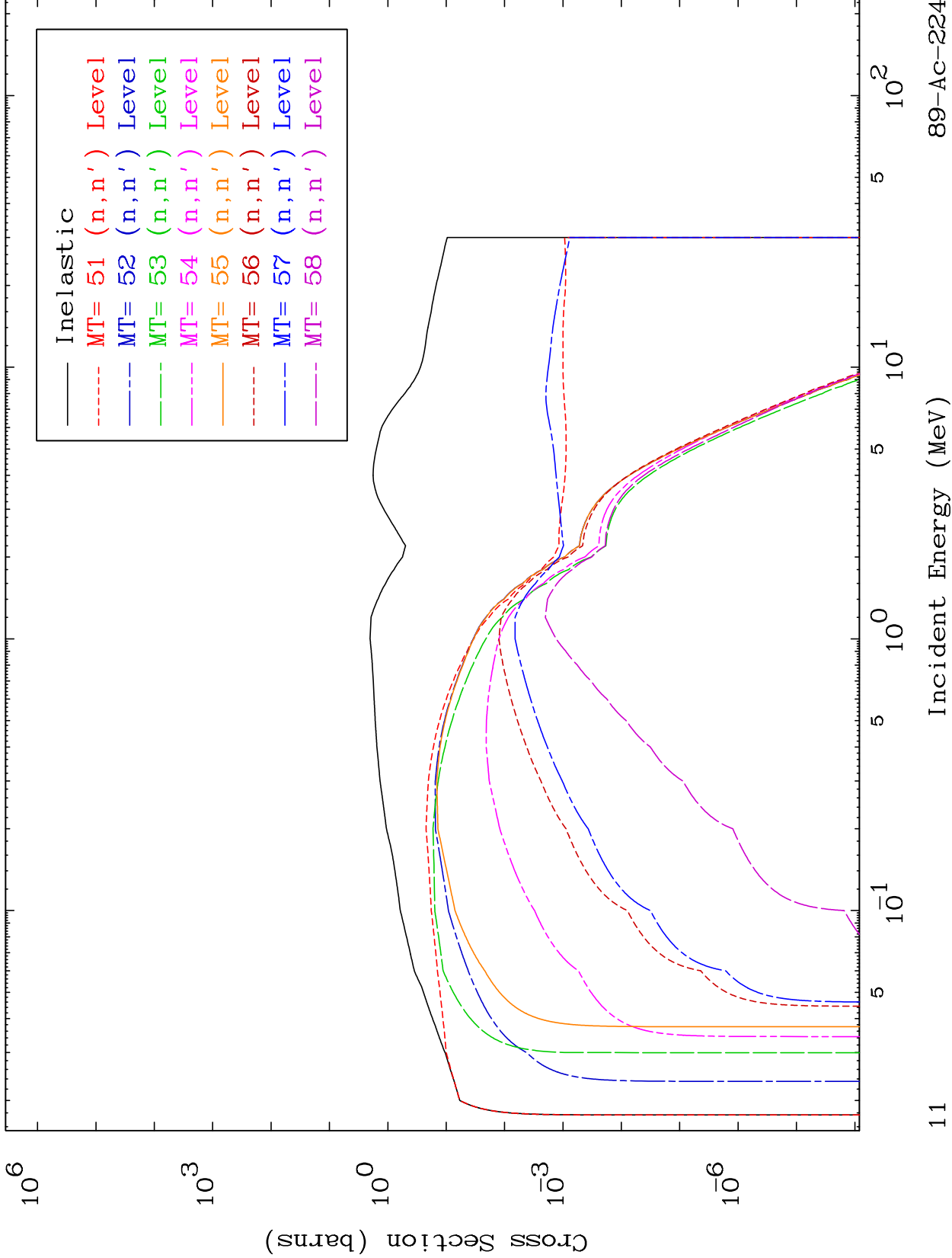
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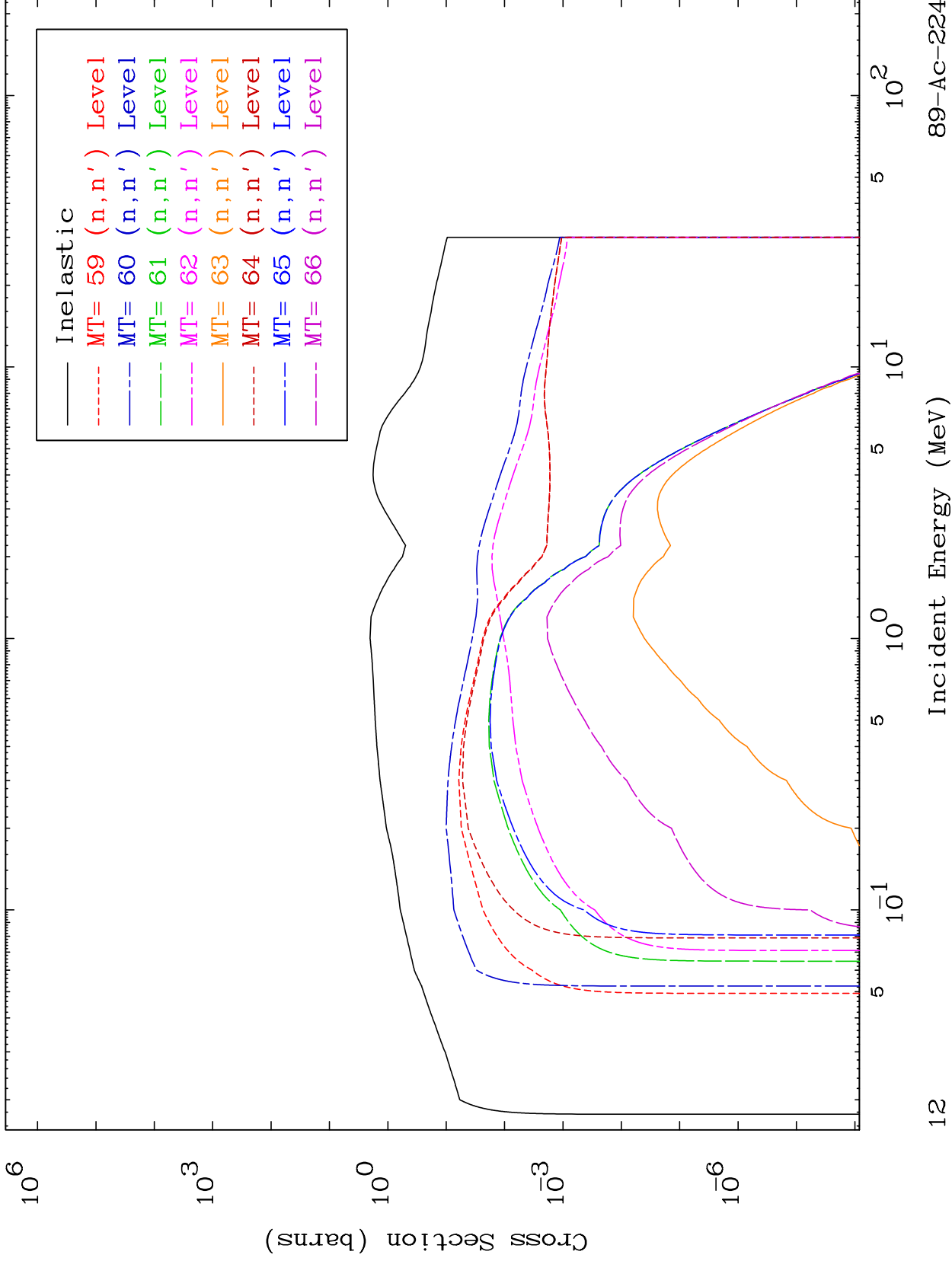
10

Incident Energy (MeV)

89-Ac-224



293 Kelvin Cross Sections

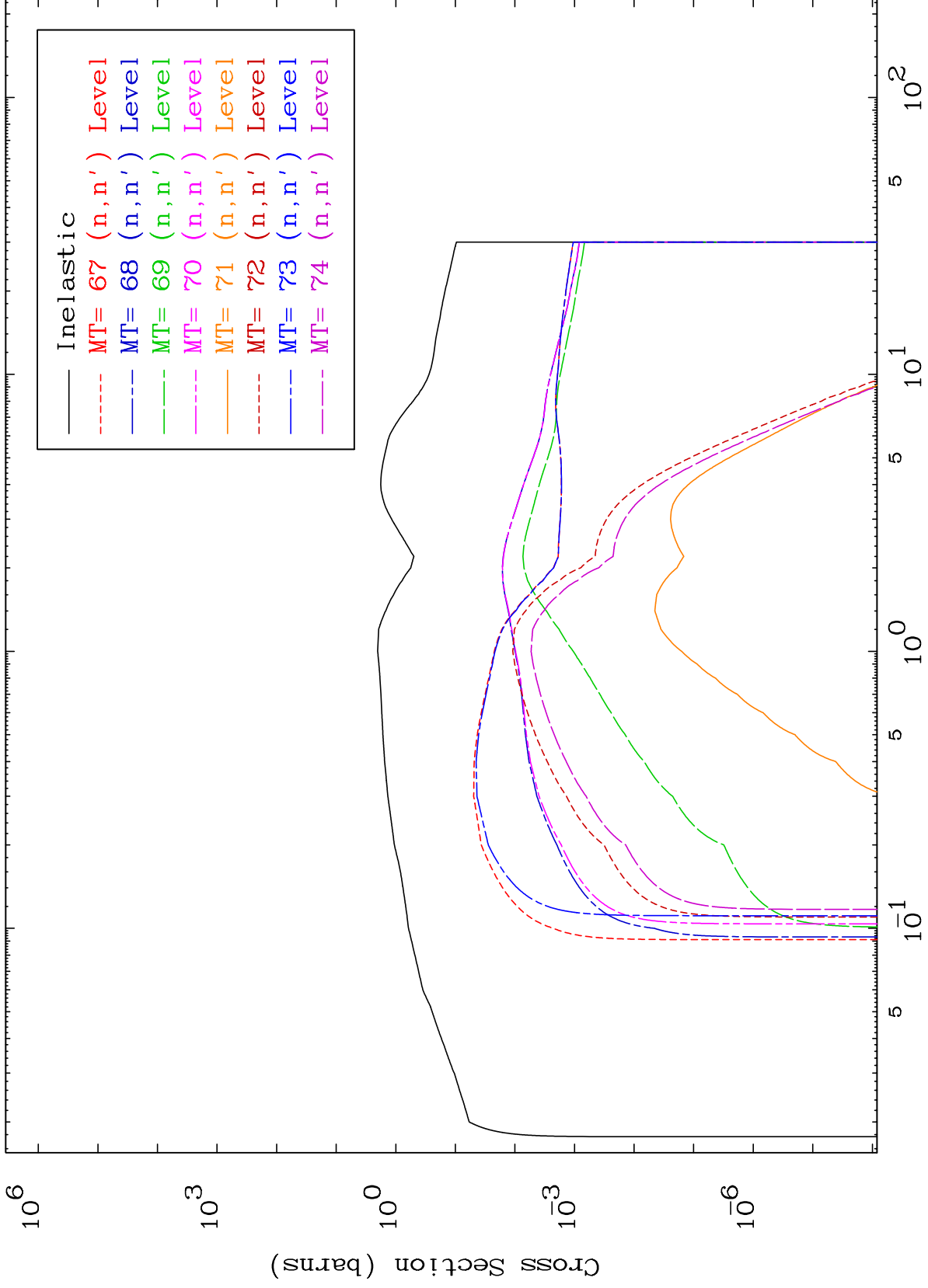


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(n,n') Levels

89-Ac-224

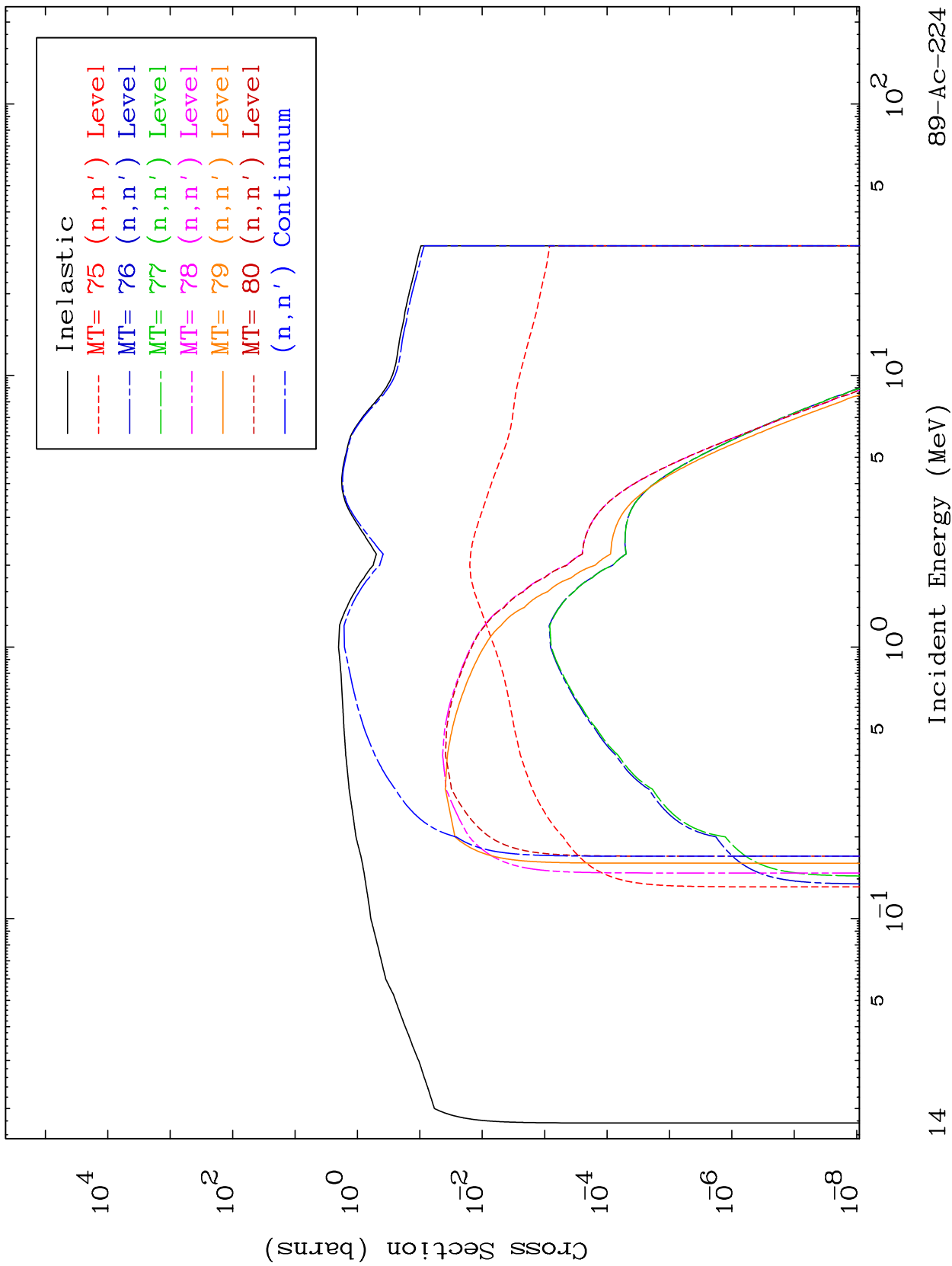
293 Kelvin Cross Sections



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Incident Energy (MeV)

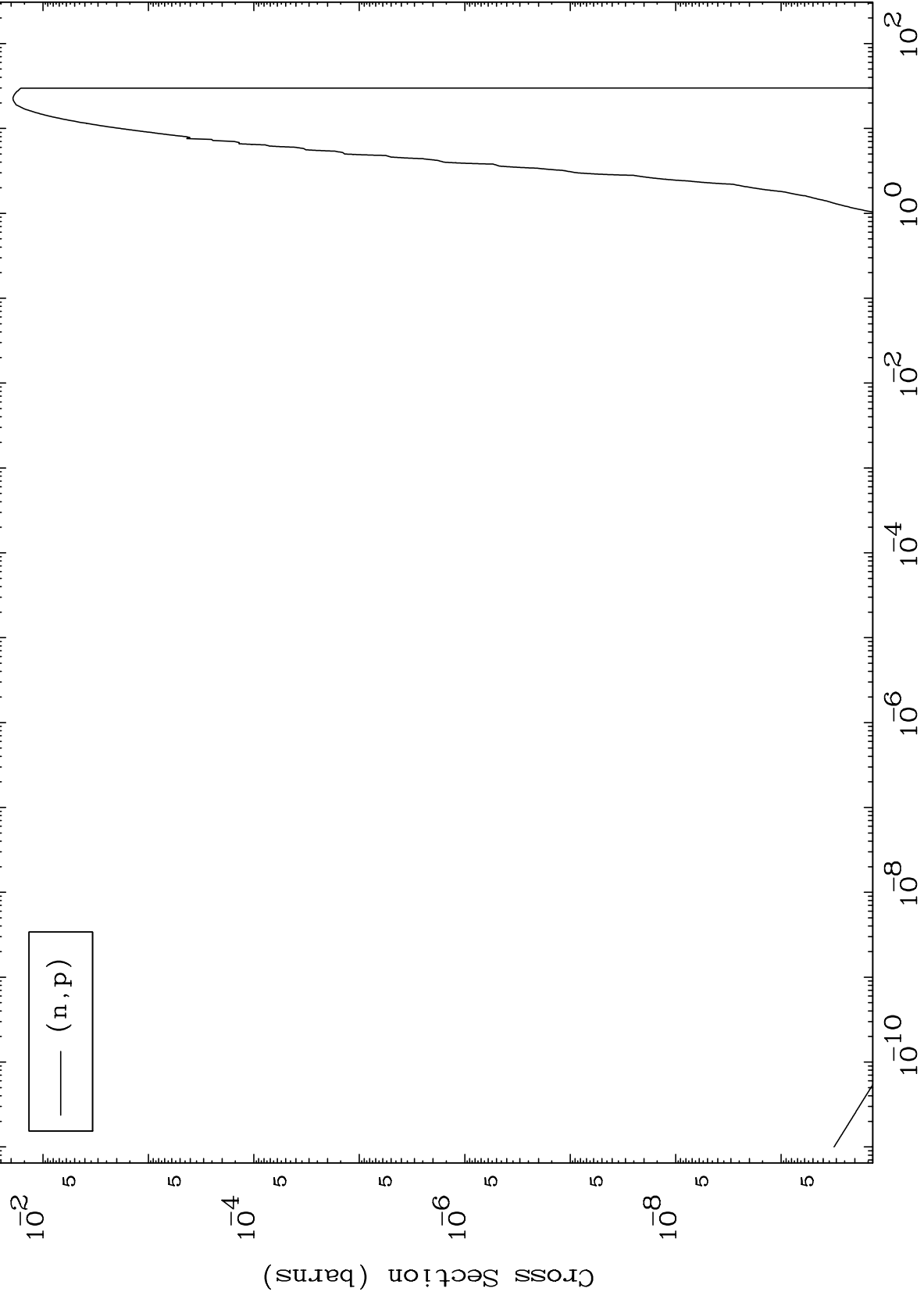
89-Ac-224



MAT 8922

(n,p) Levels
293 Kelvin Cross Sections

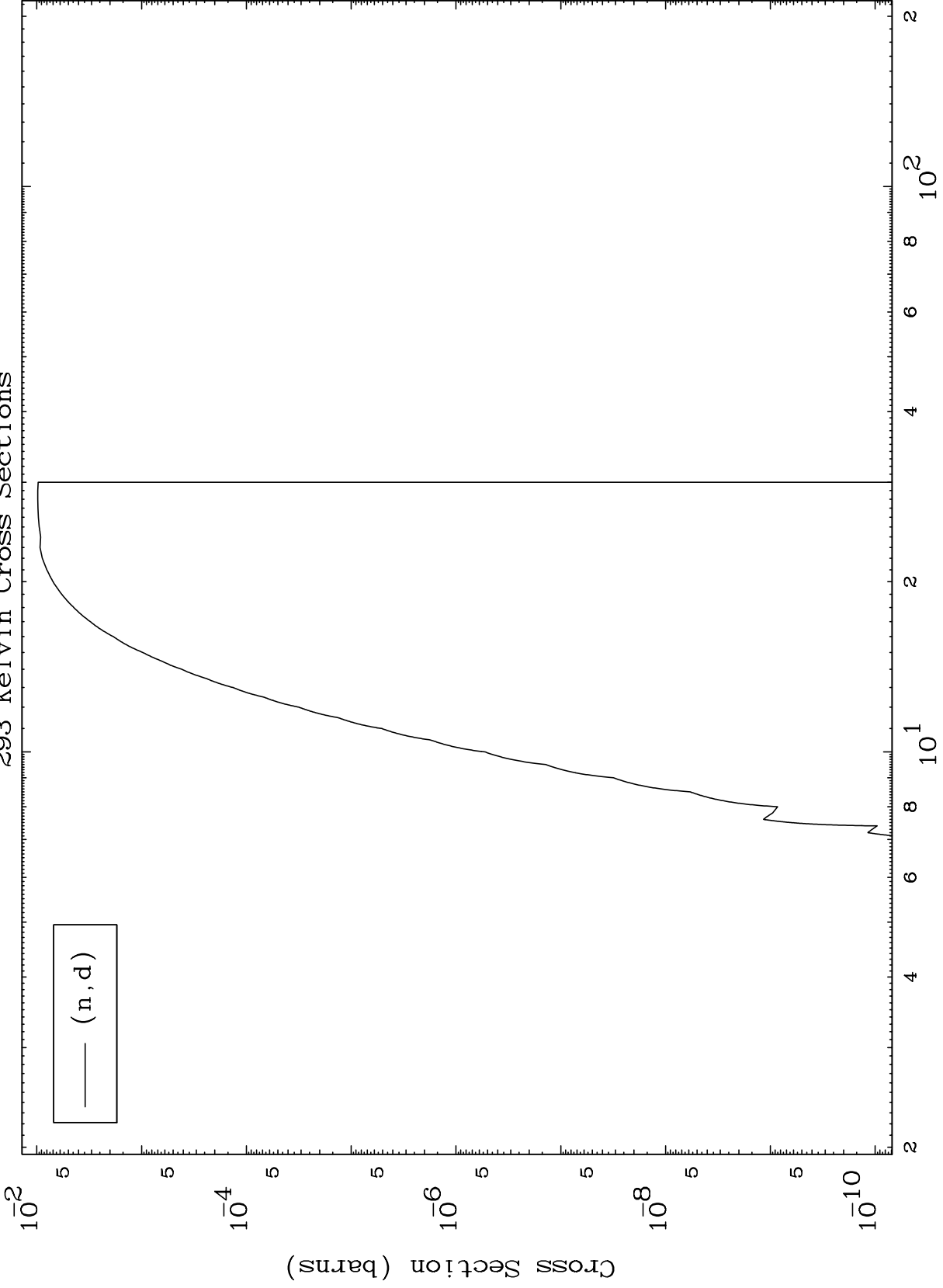
89-Ac-224



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(n,d) Levels
293 Kelvin Cross Sections

89-Ac-224



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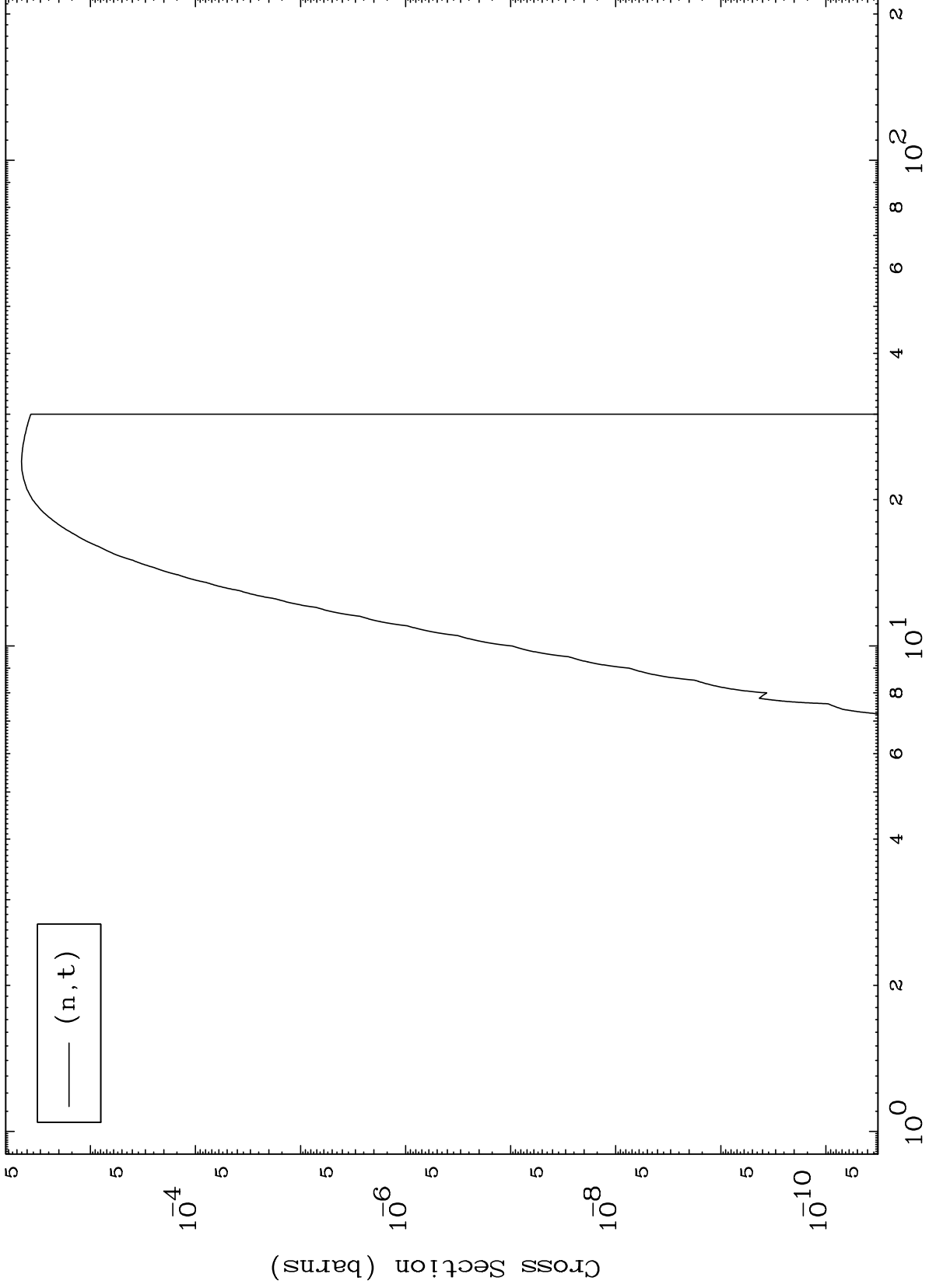
Incident Energy (MeV)

89-Ac-224

MAT 89222

(n,t) Levels
293 Kelvin Cross Sections

89-Ac-224



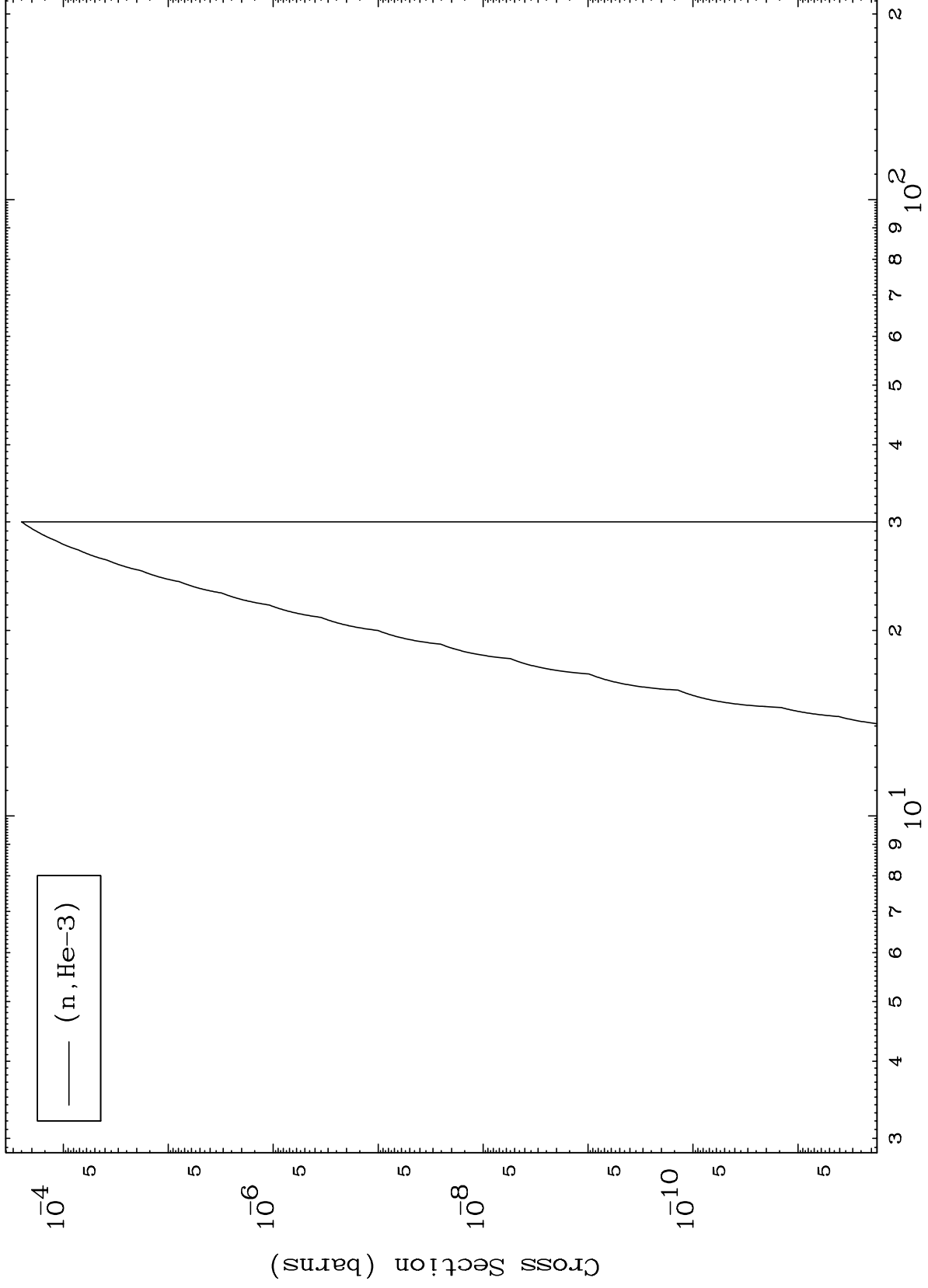
Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,He3) Levels
293 Kelvin Cross Sections

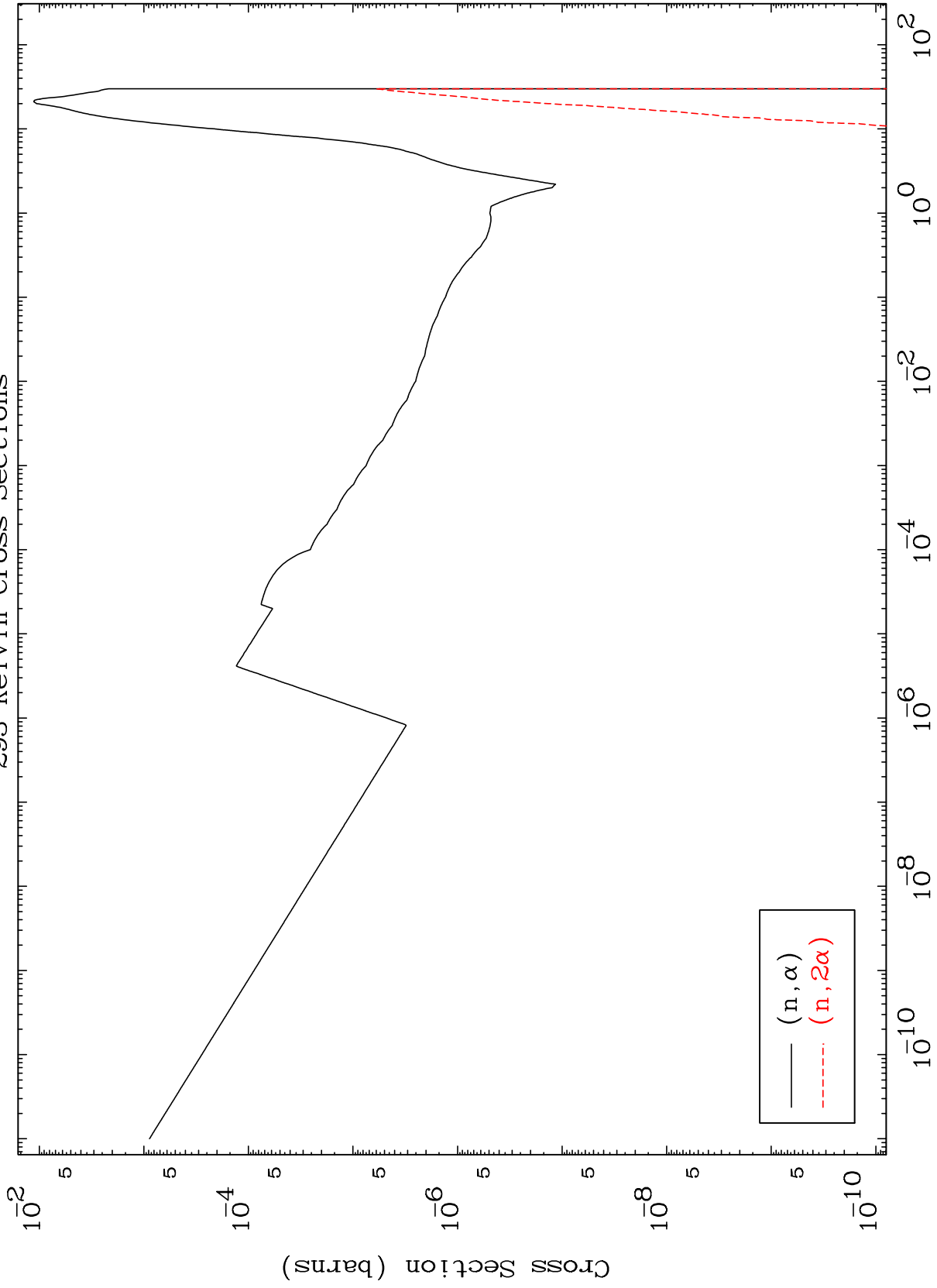
89-Ac-224



MAT 8922

(n, α) Levels
293 Kelvin Cross Sections

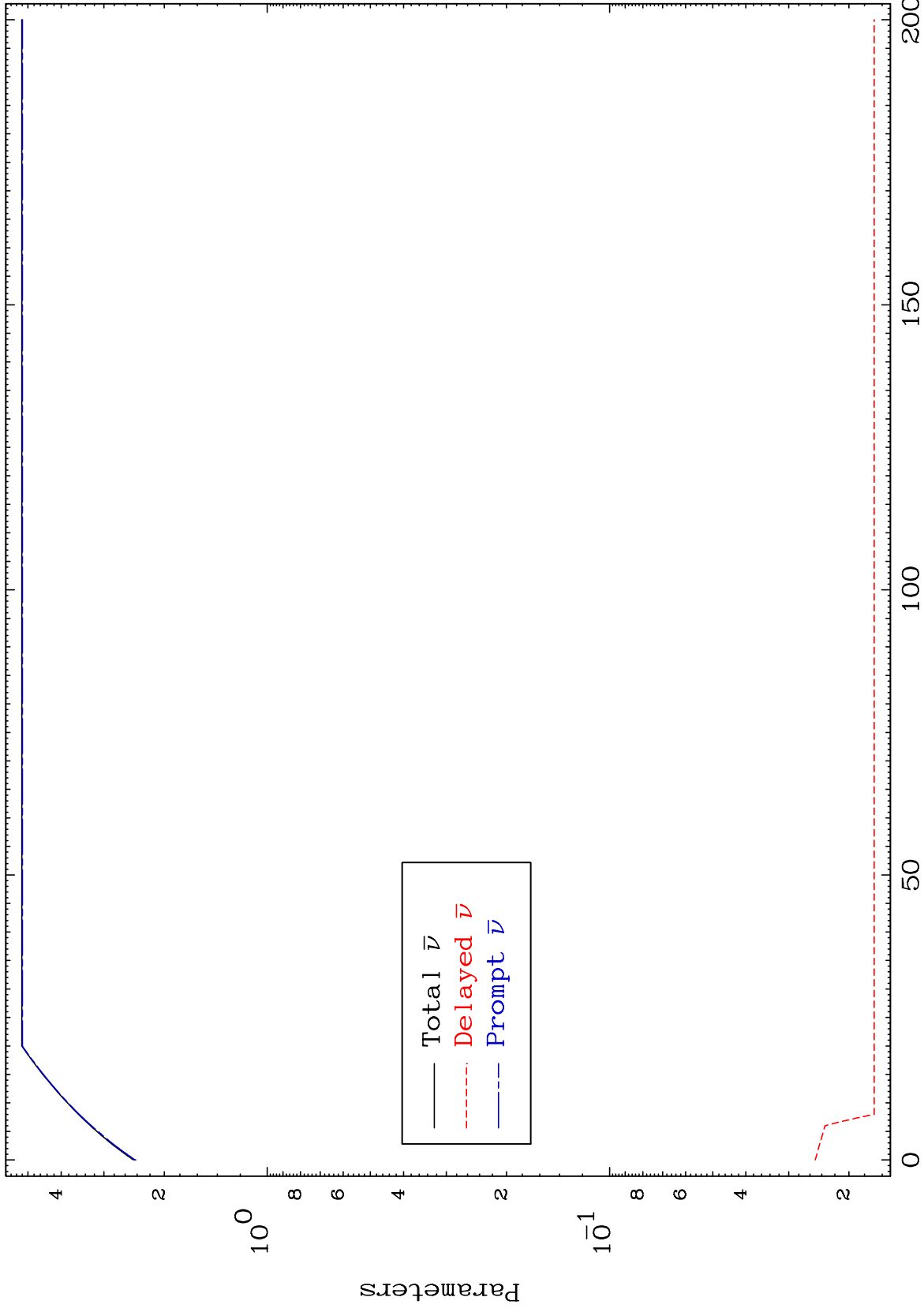
89-Ac-224



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Energy Release
Parameters

89-Ac-224



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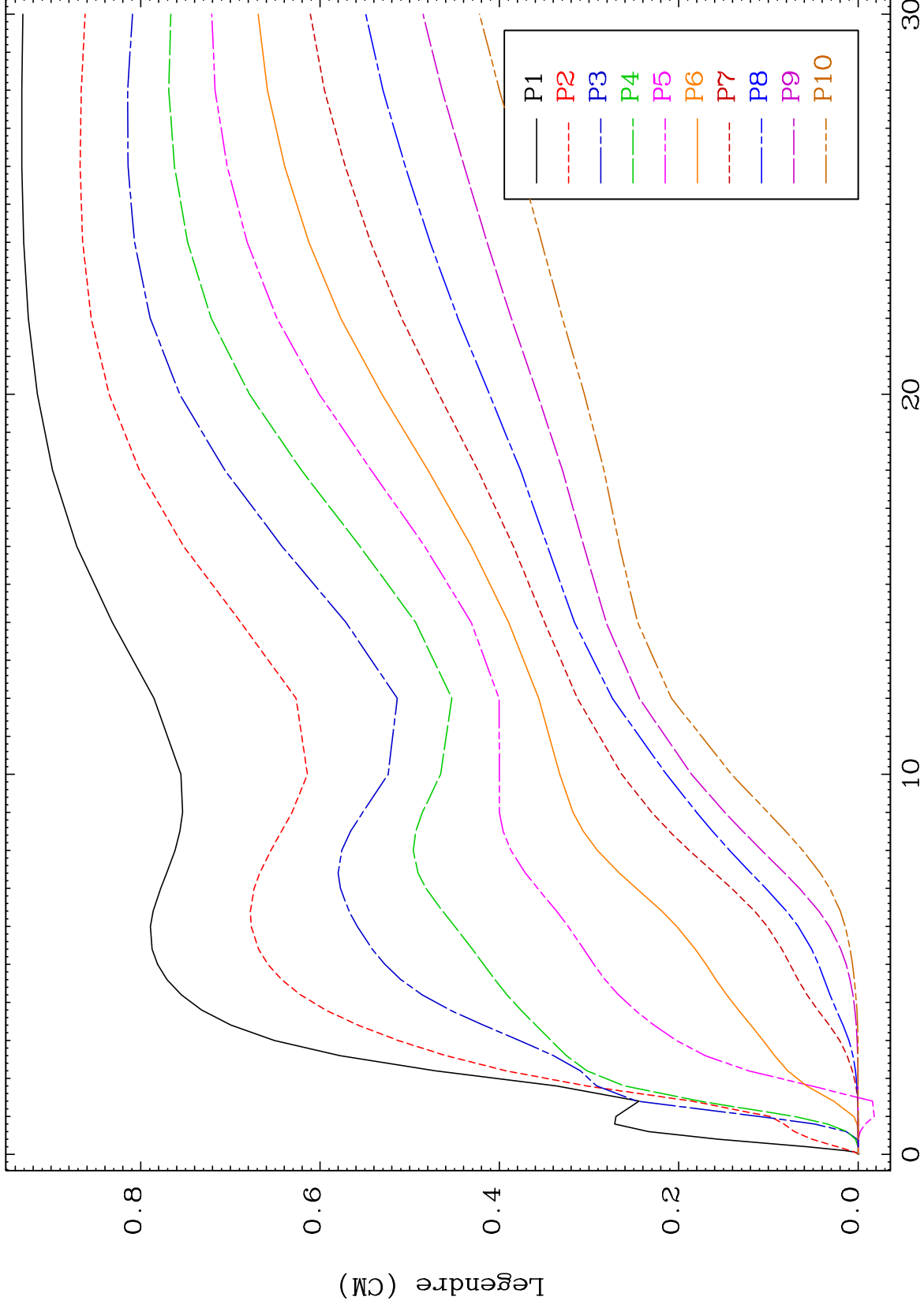
Incident Energy (MeV)

89-Ac-224

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89-Ac-224

Elastic Legendre Coefficients



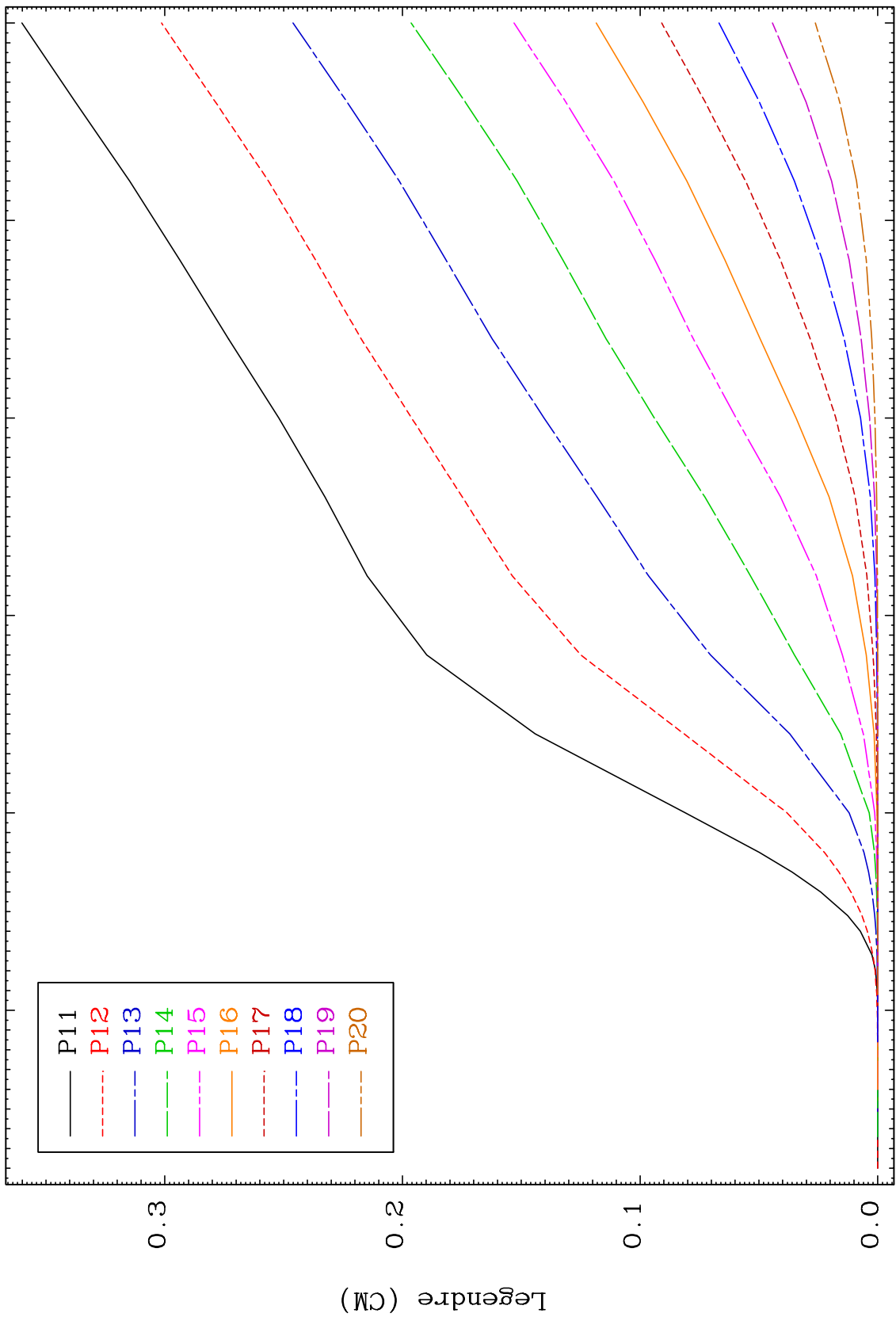
21

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Elastic Legendre Coefficients

89-Ac-224



22

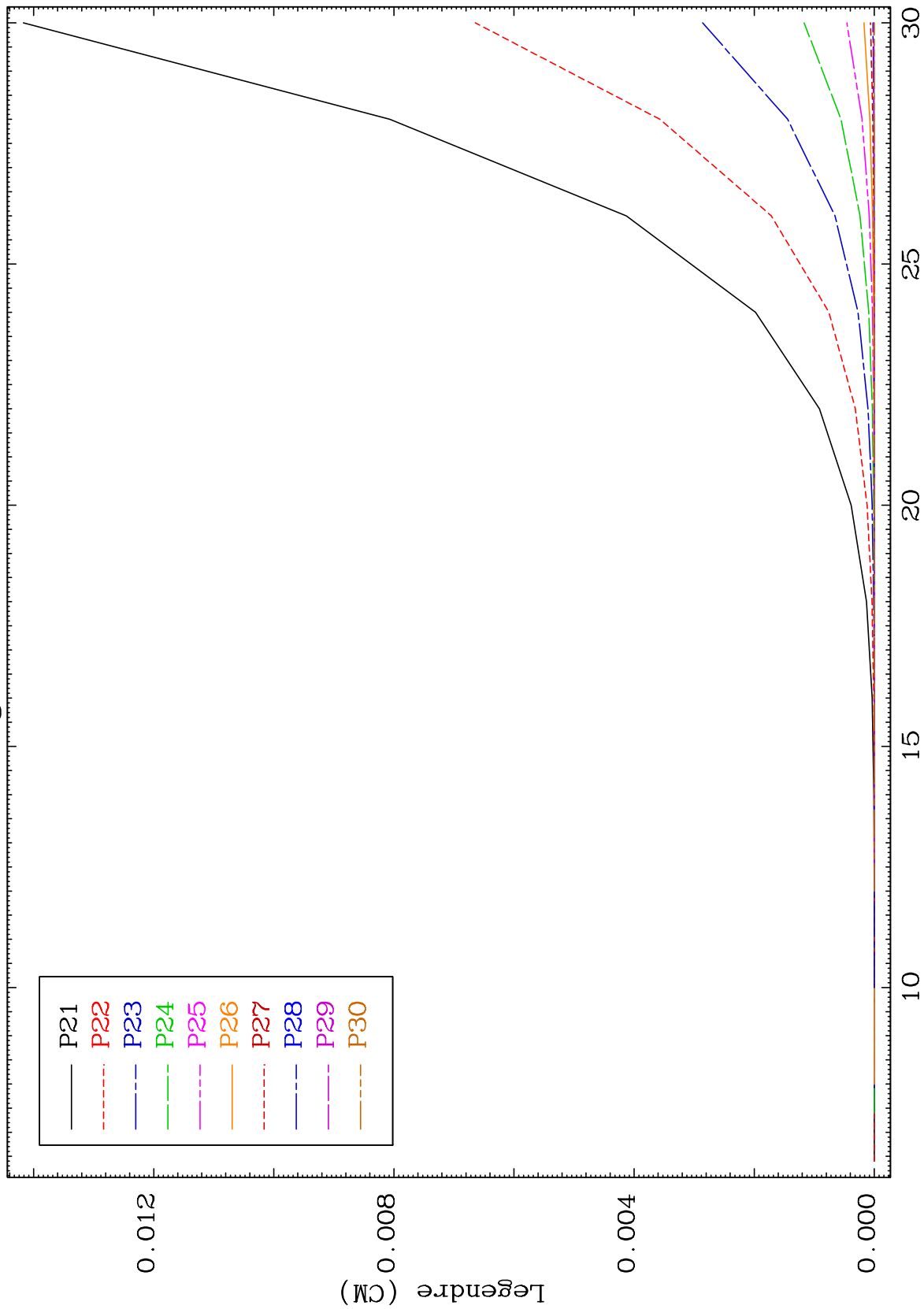
Incident Energy (MeV)

89-Ac-224

MAT 8922

Elastic Legendre Coefficients

89-Ac-224



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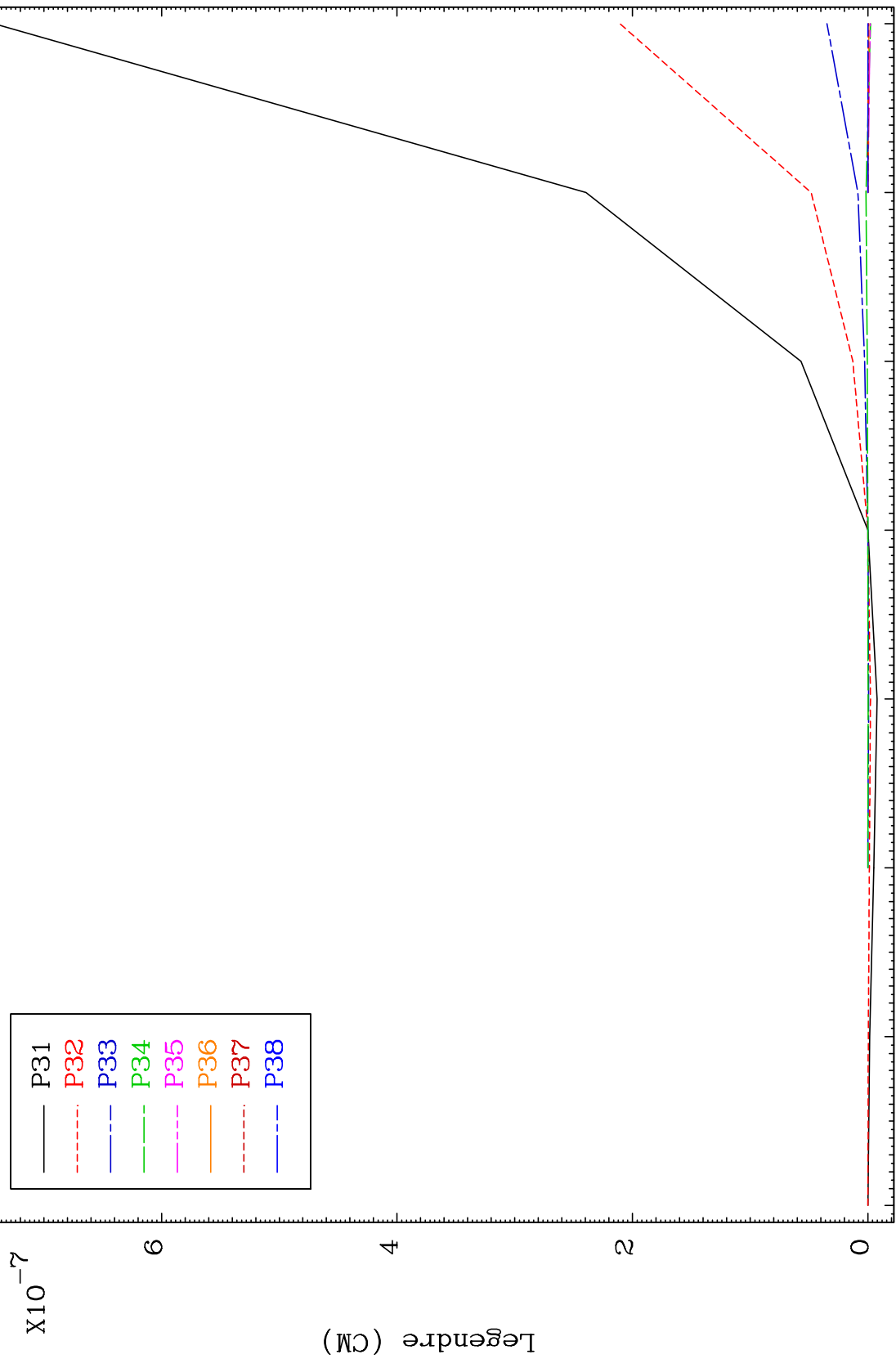
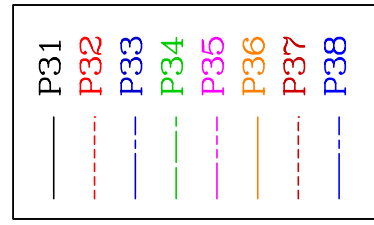
89-Ac-224

Incident Energy (MeV)

MAT 8922

Elastic Legendre Coefficients

89-Ac-224

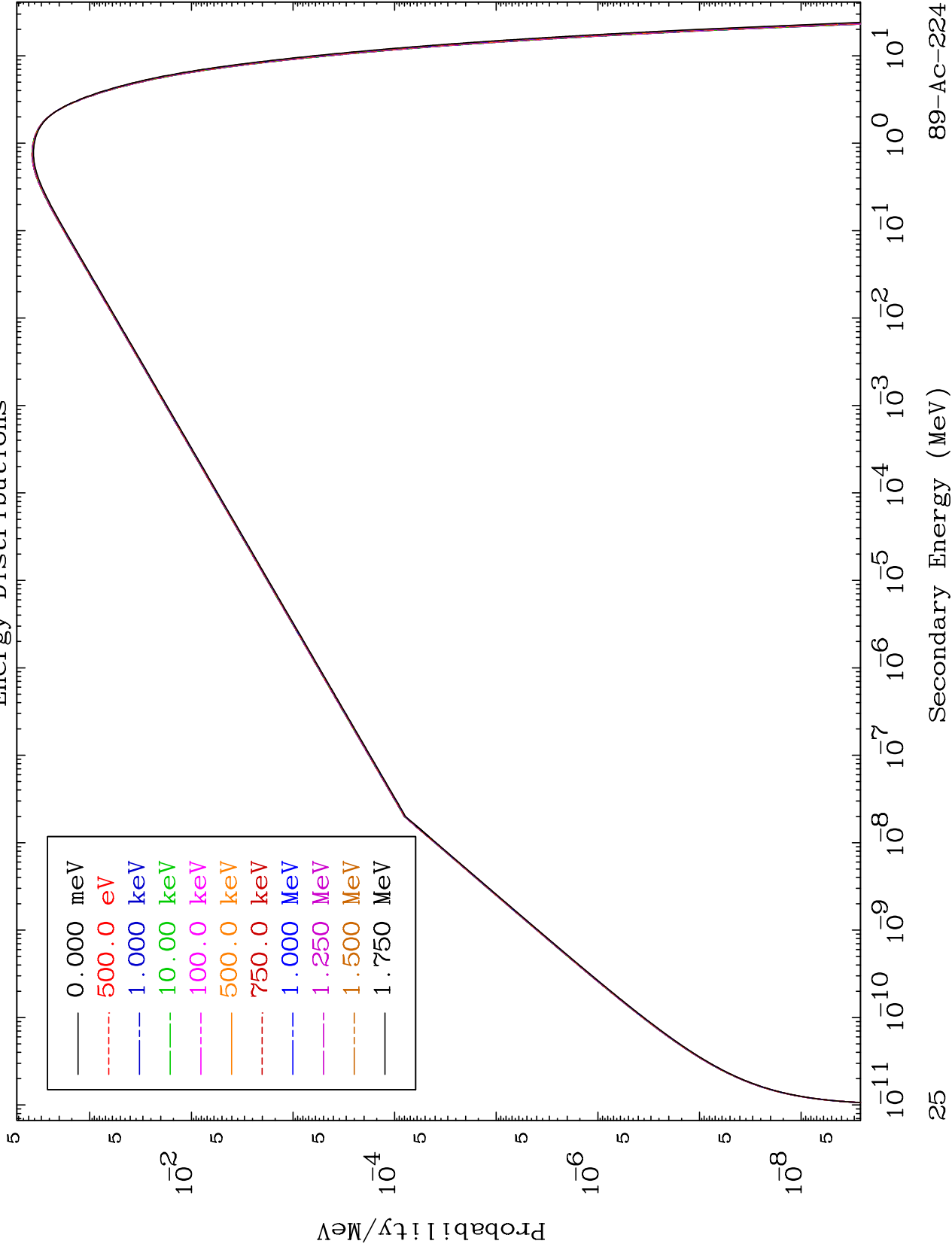


24

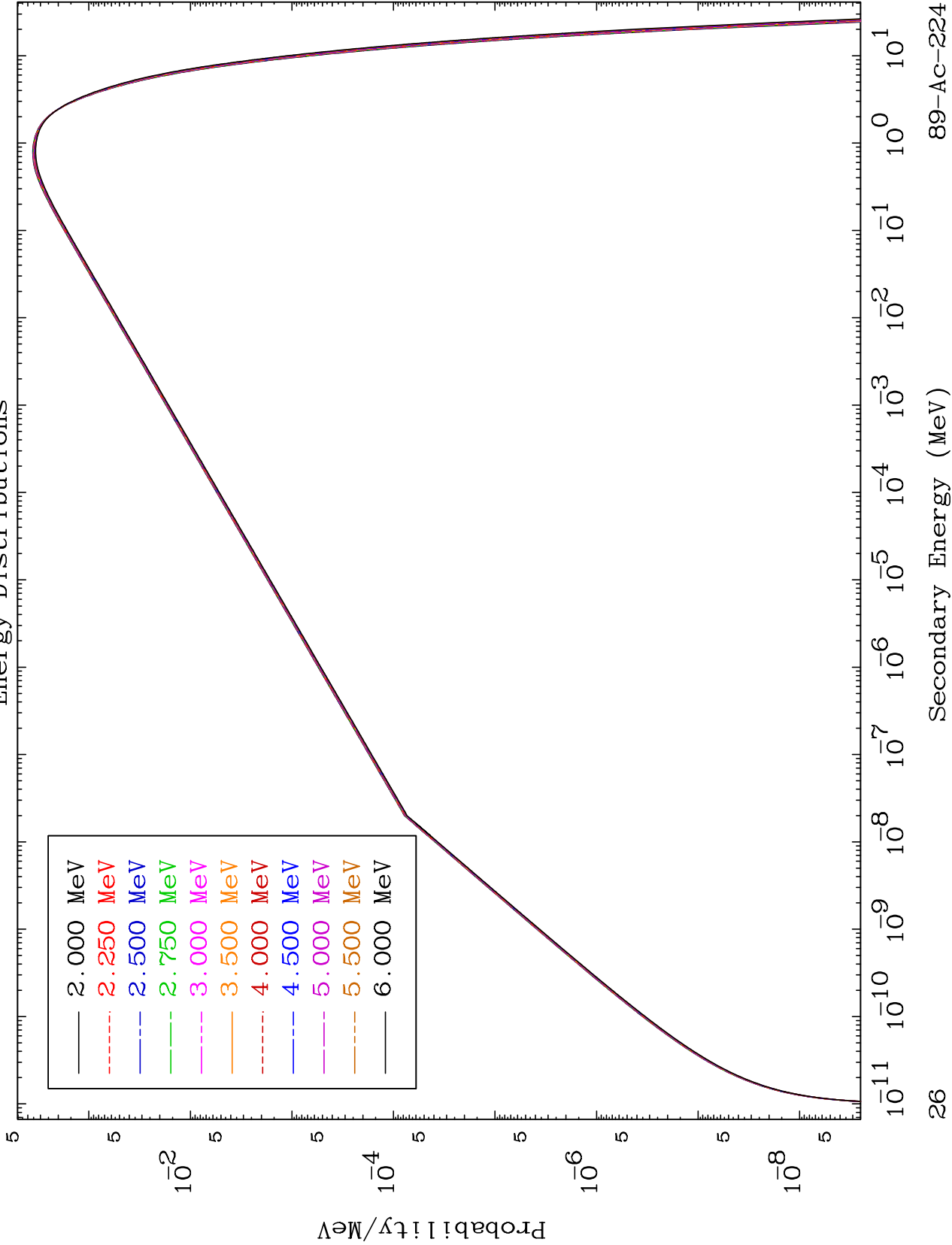
Incident Energy (MeV)

89-Ac-224

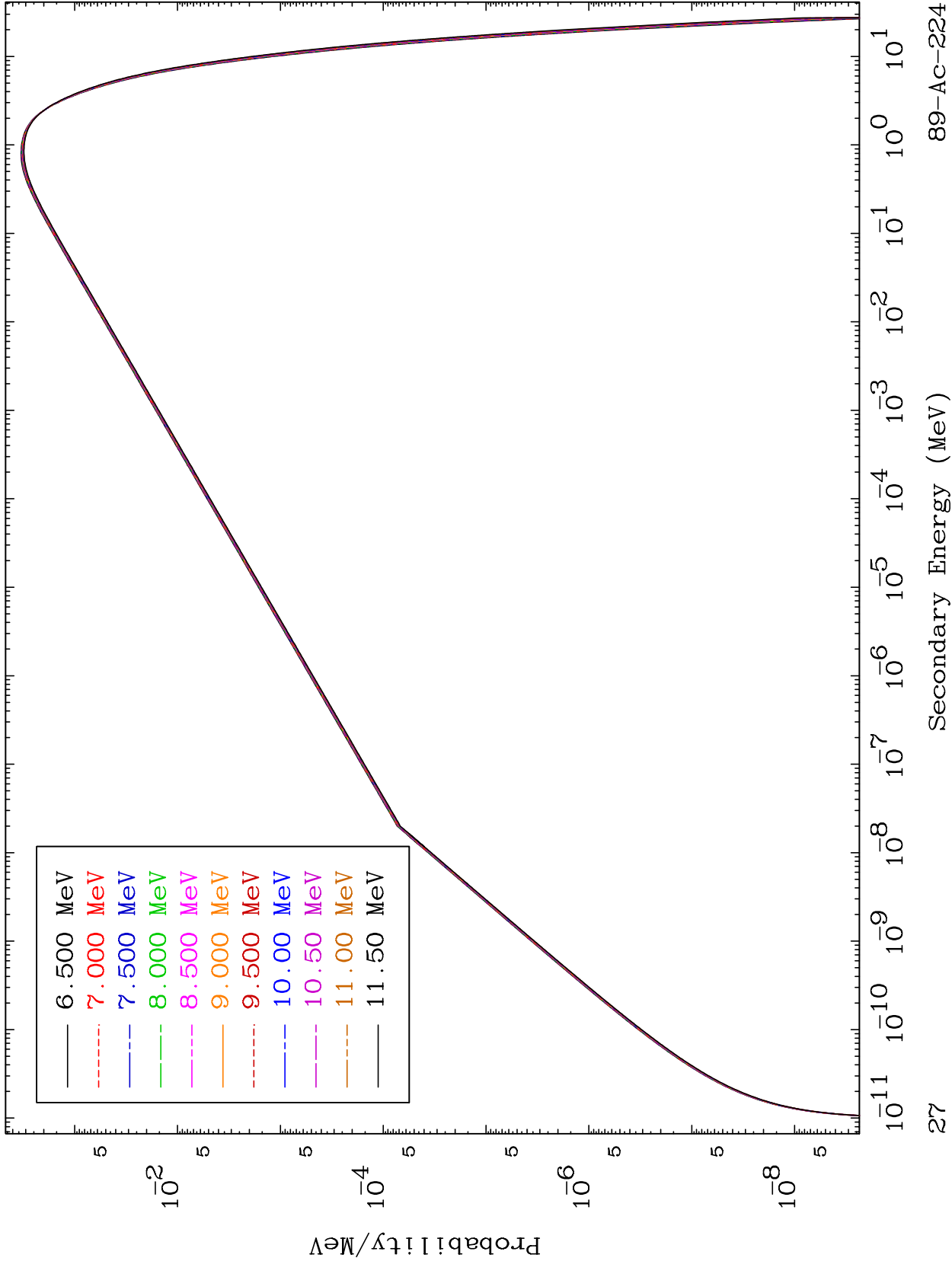
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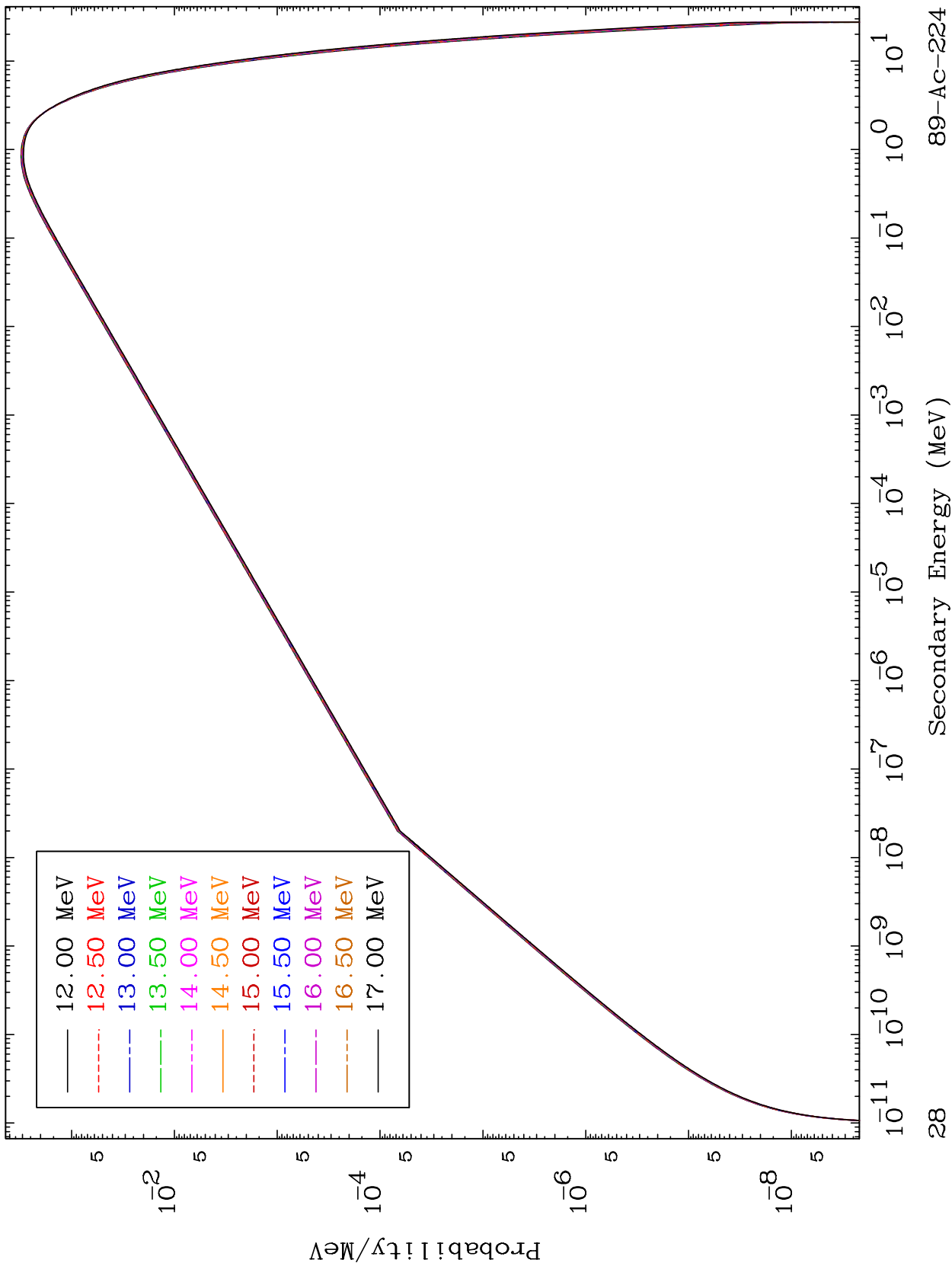
Fission Energy Distributions



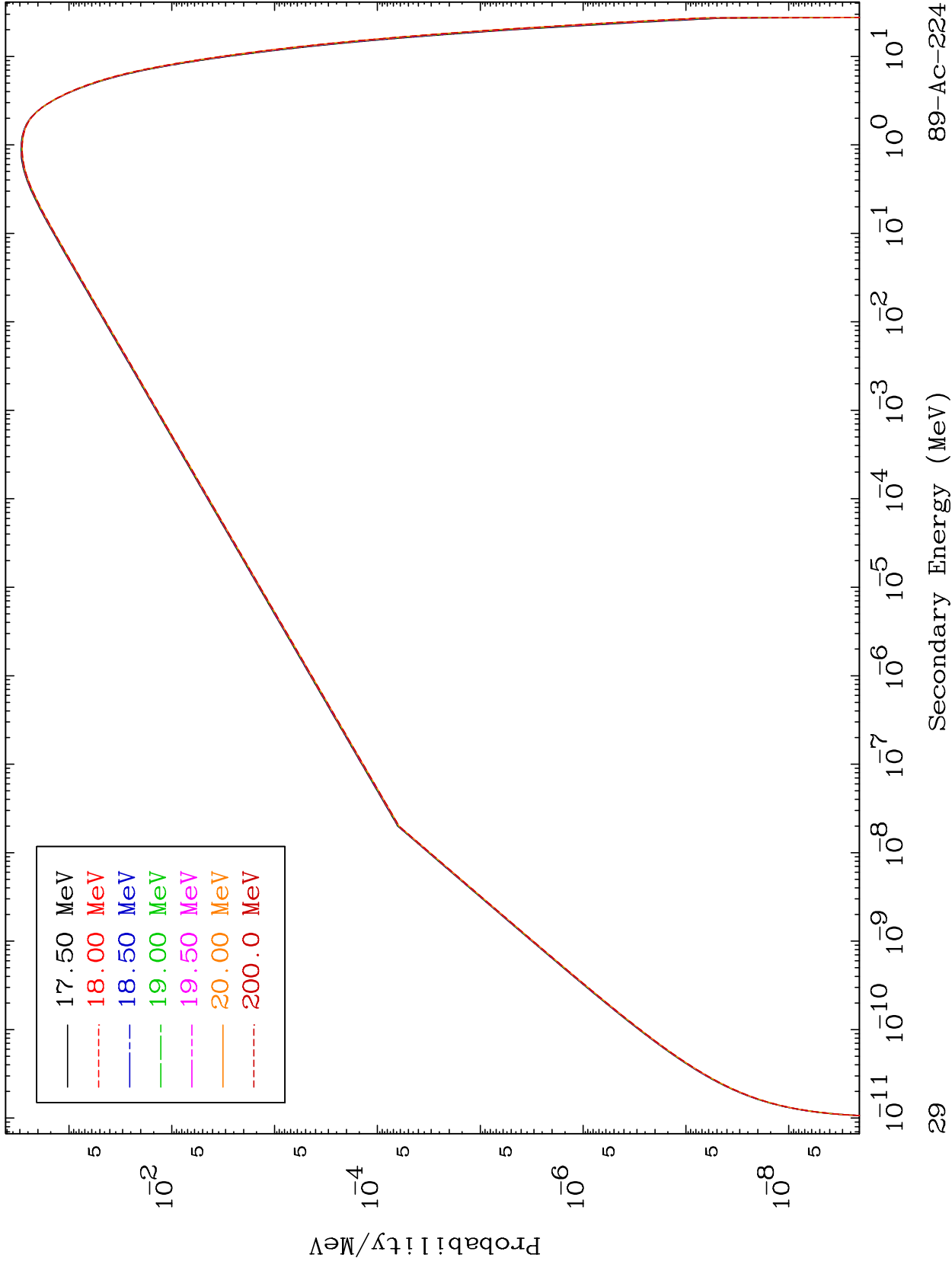
Fission Energy Distributions



Fission Energy Distributions



Fission Energy Distributions

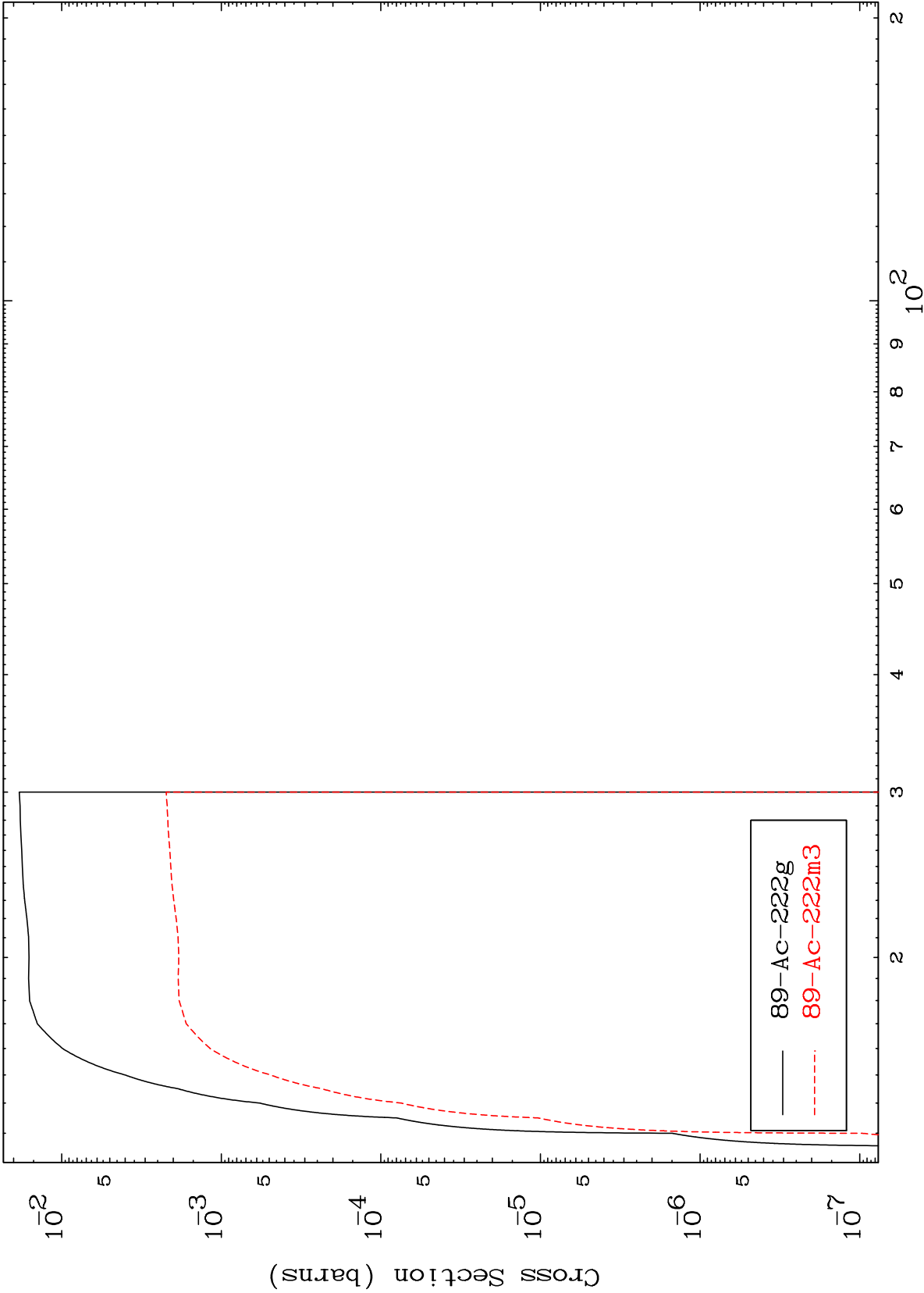


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(n,3n)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

89-Ac-224