

Program EVALPLOT
(Version 2021-1)

by

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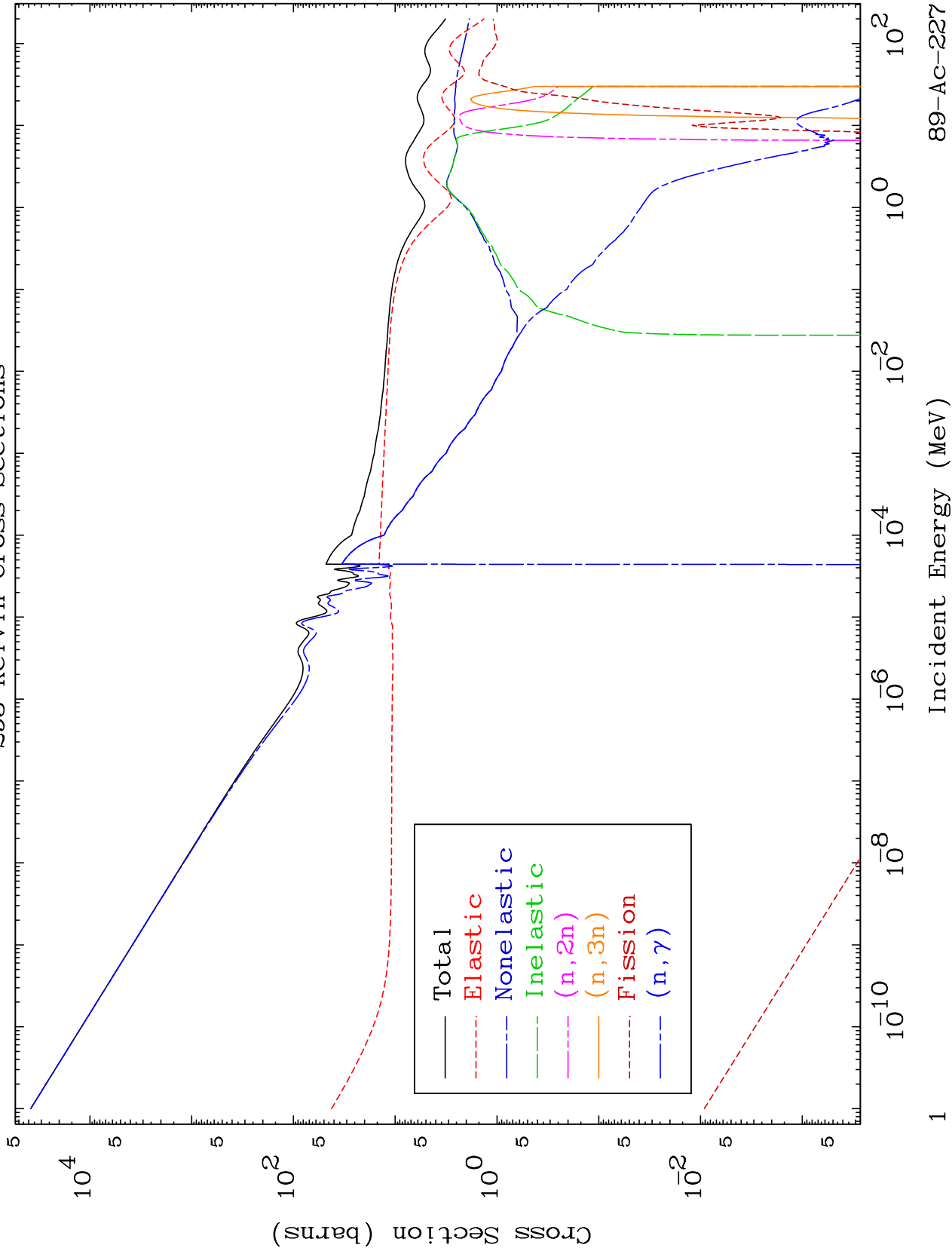
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Press Mouse Button to Start

MAT 8931

Neutron Major
293 Kelvin Cross Sections

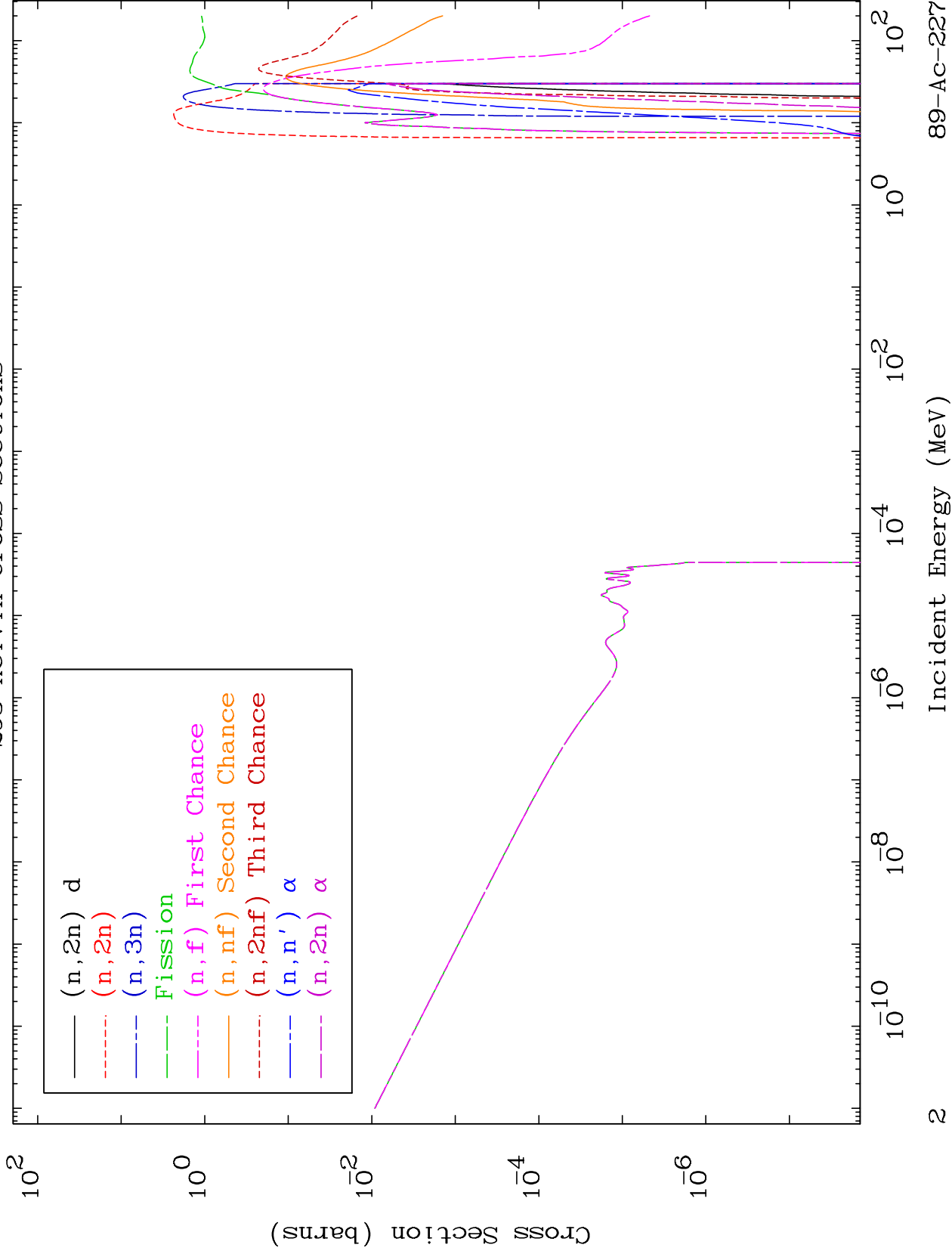
89-Ac-227



MAT 8931

Neutron Absorption
293 Kelvin Cross Sections

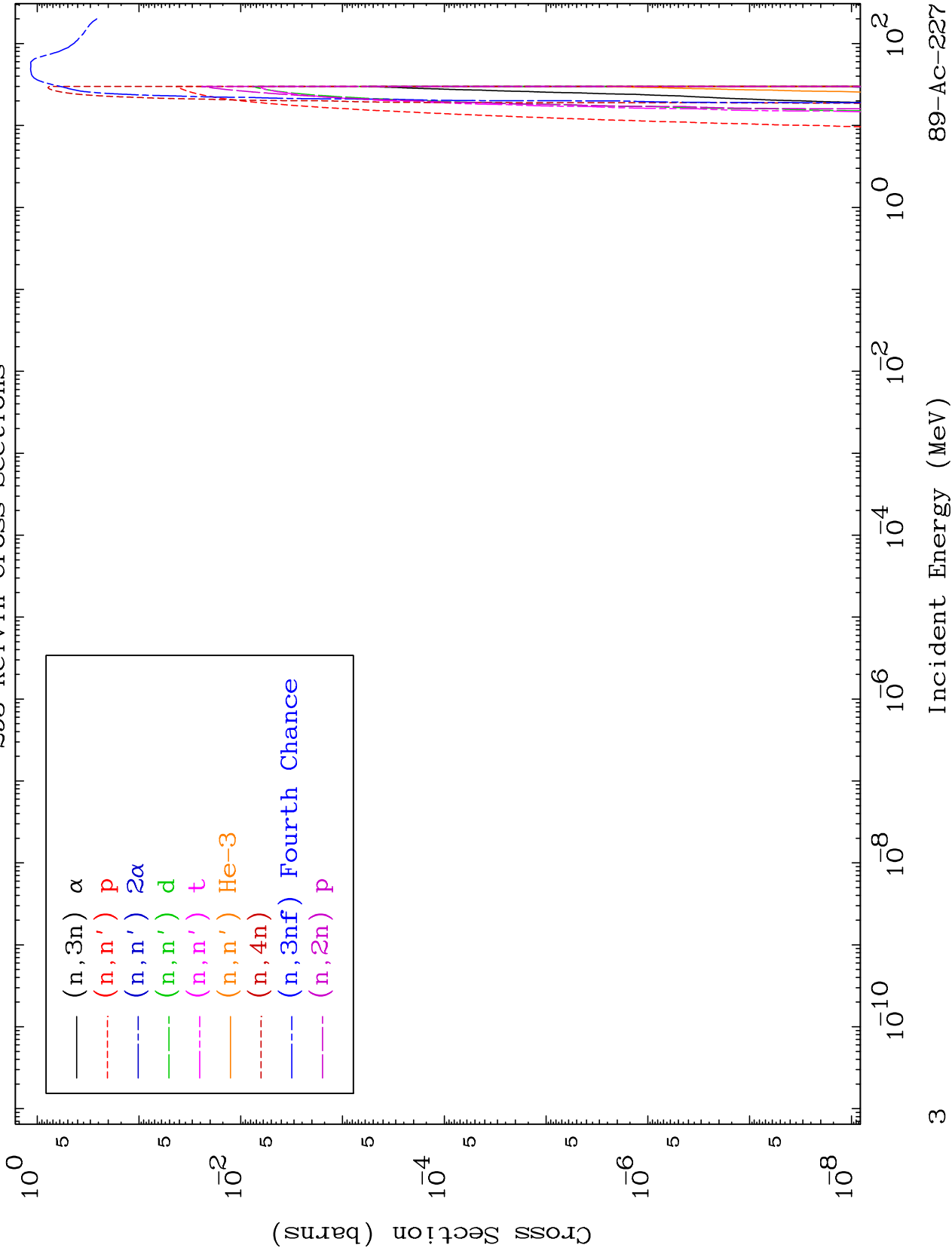
89-Ac-227



MAT 8931

Neutron Absorption
293 Kelvin Cross Sections

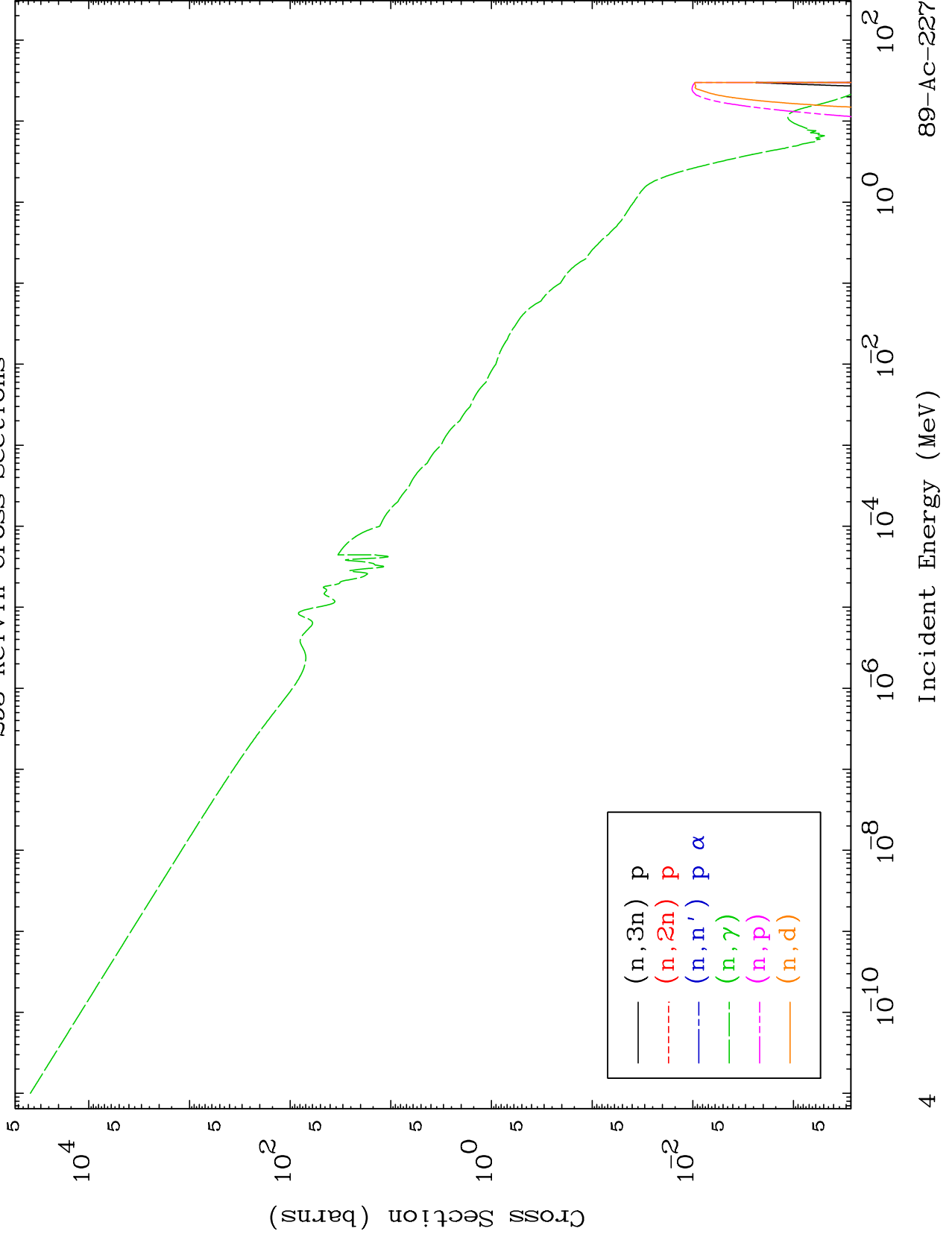
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MAT 8931

Neutron Absorption
293 Kelvin Cross Sections

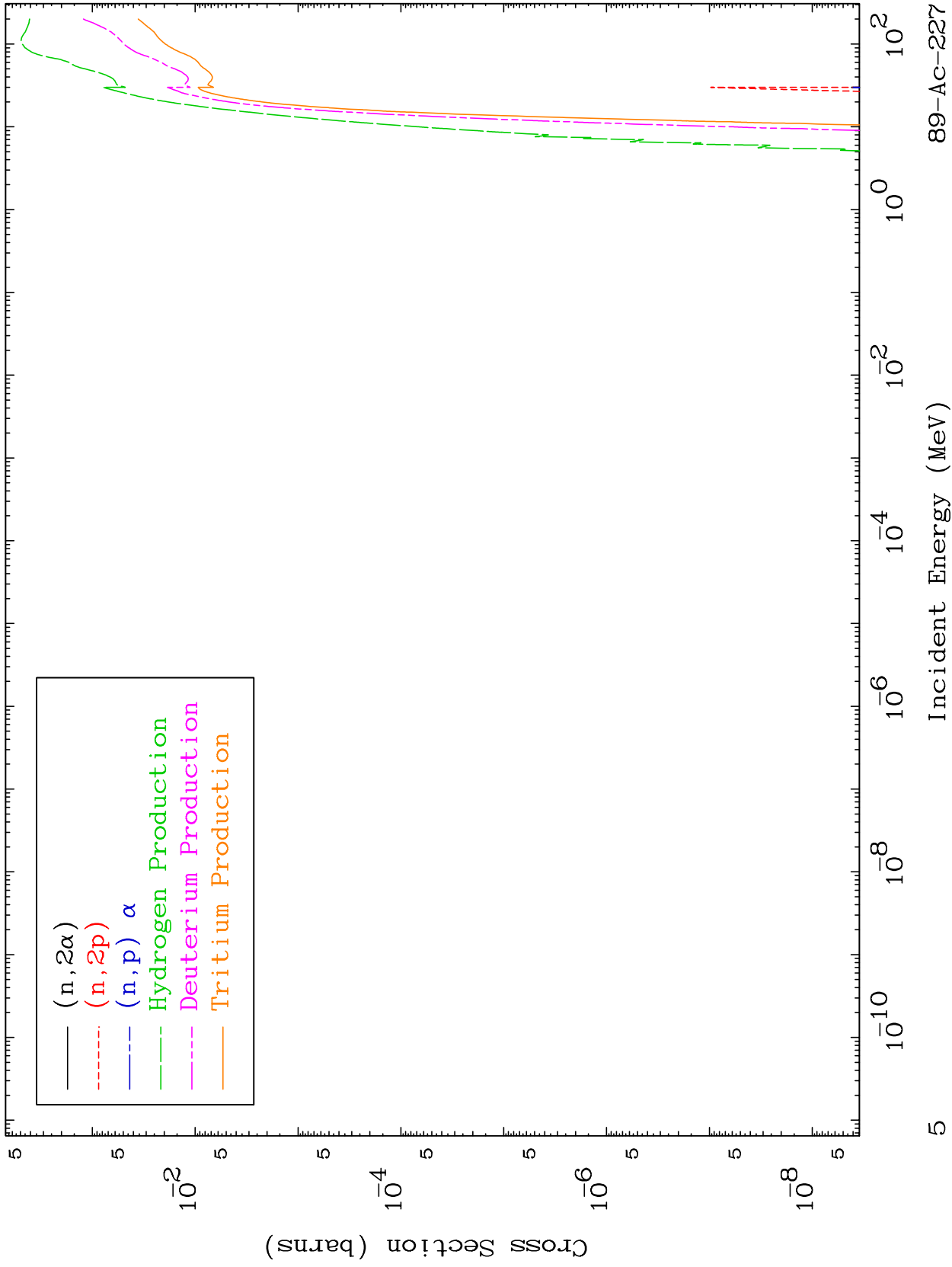
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Neutron Absorption
293 Kelvin Cross Sections

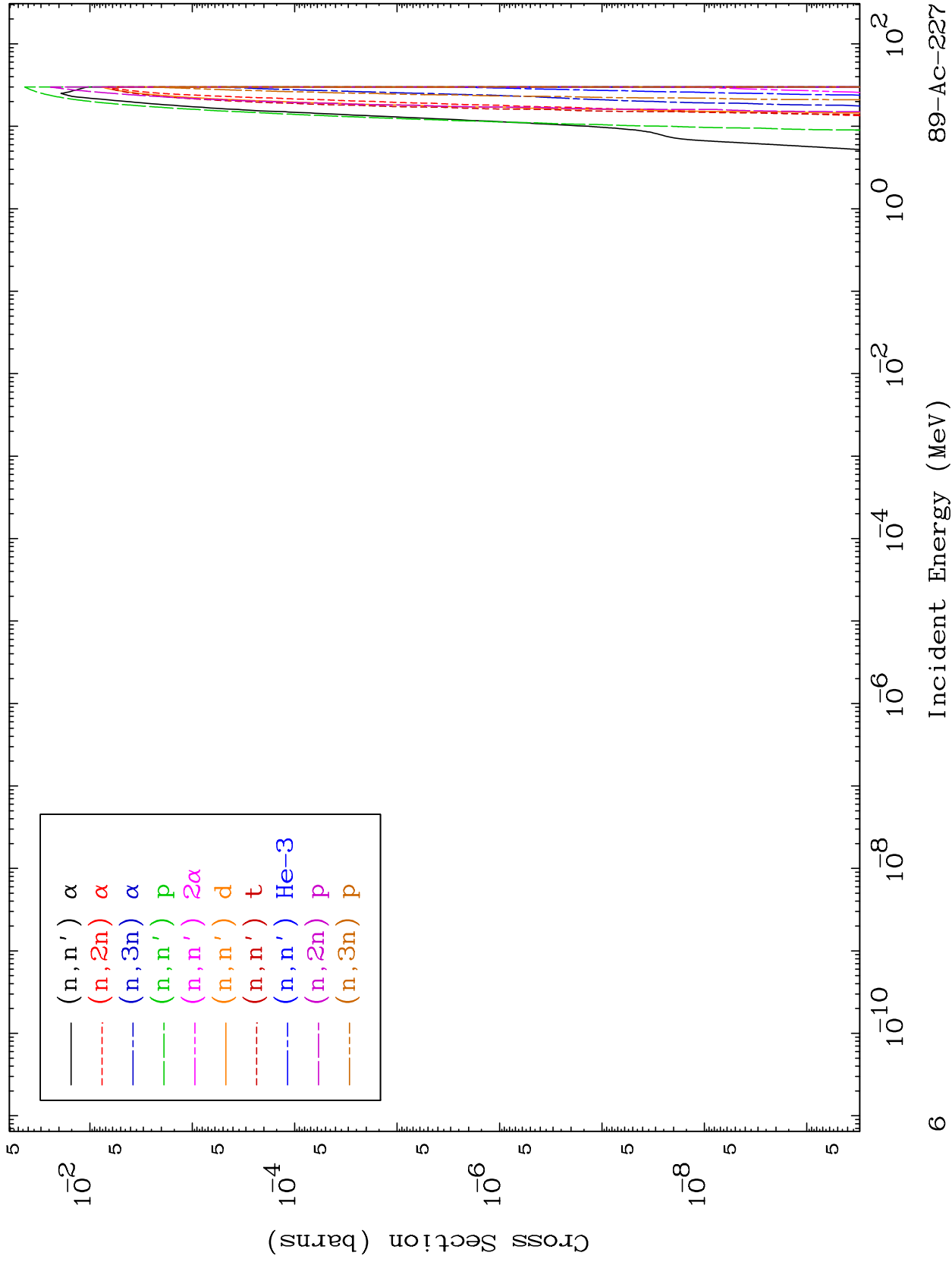
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MAT 8931

Charged Particle
293 Kelvin Cross Sections

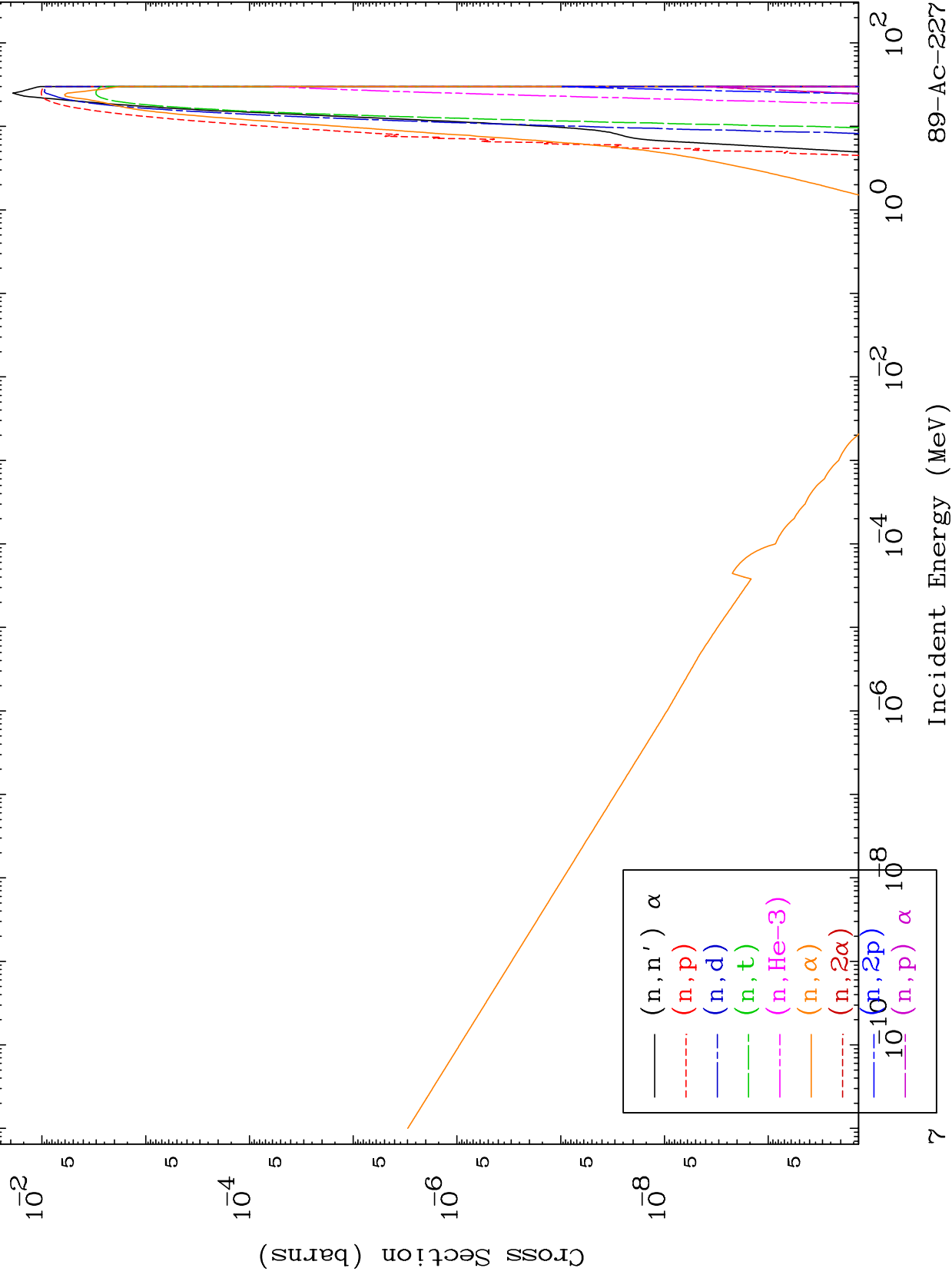
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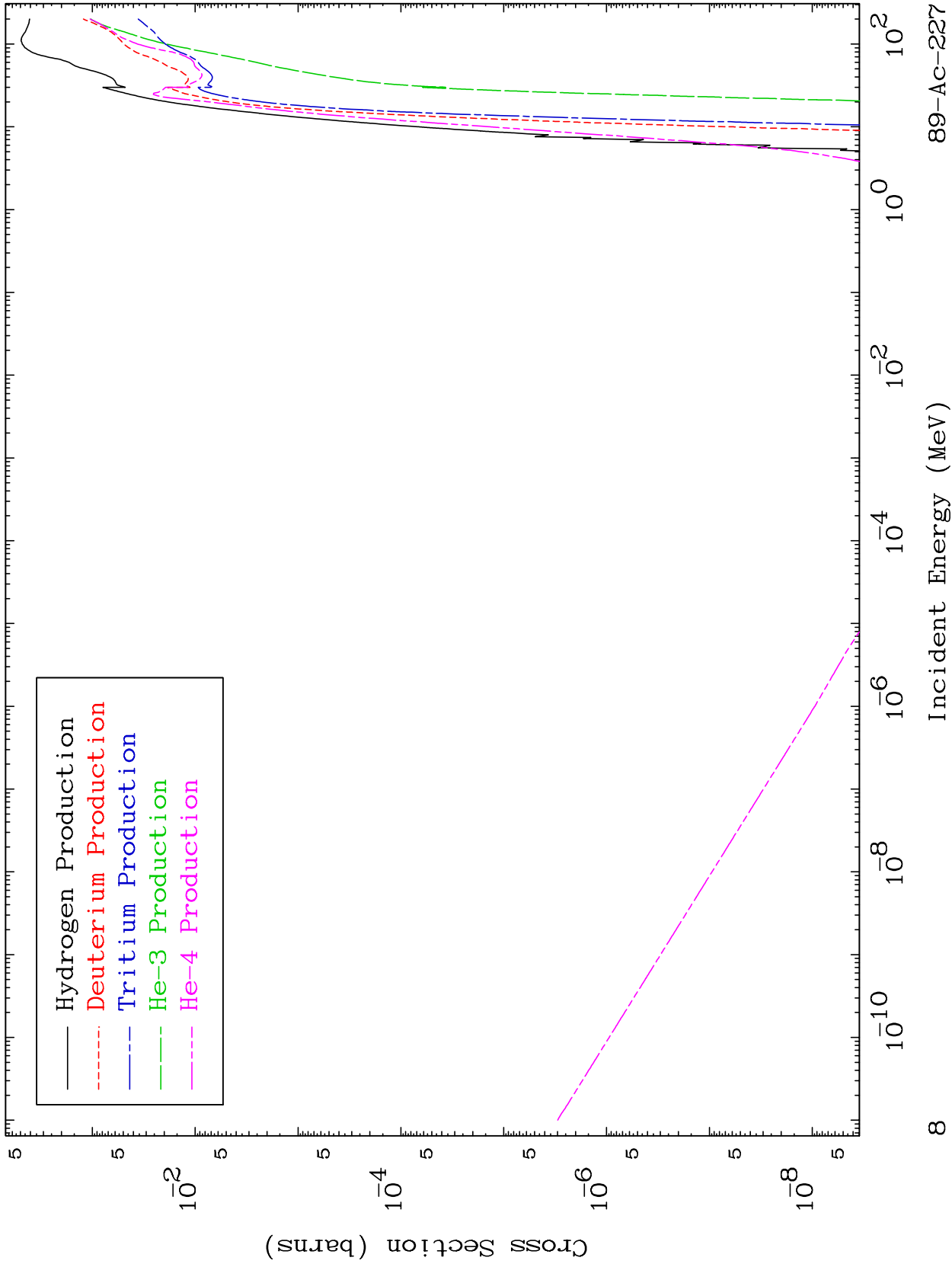


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Charged Particle
293 Kelvin Cross Sections

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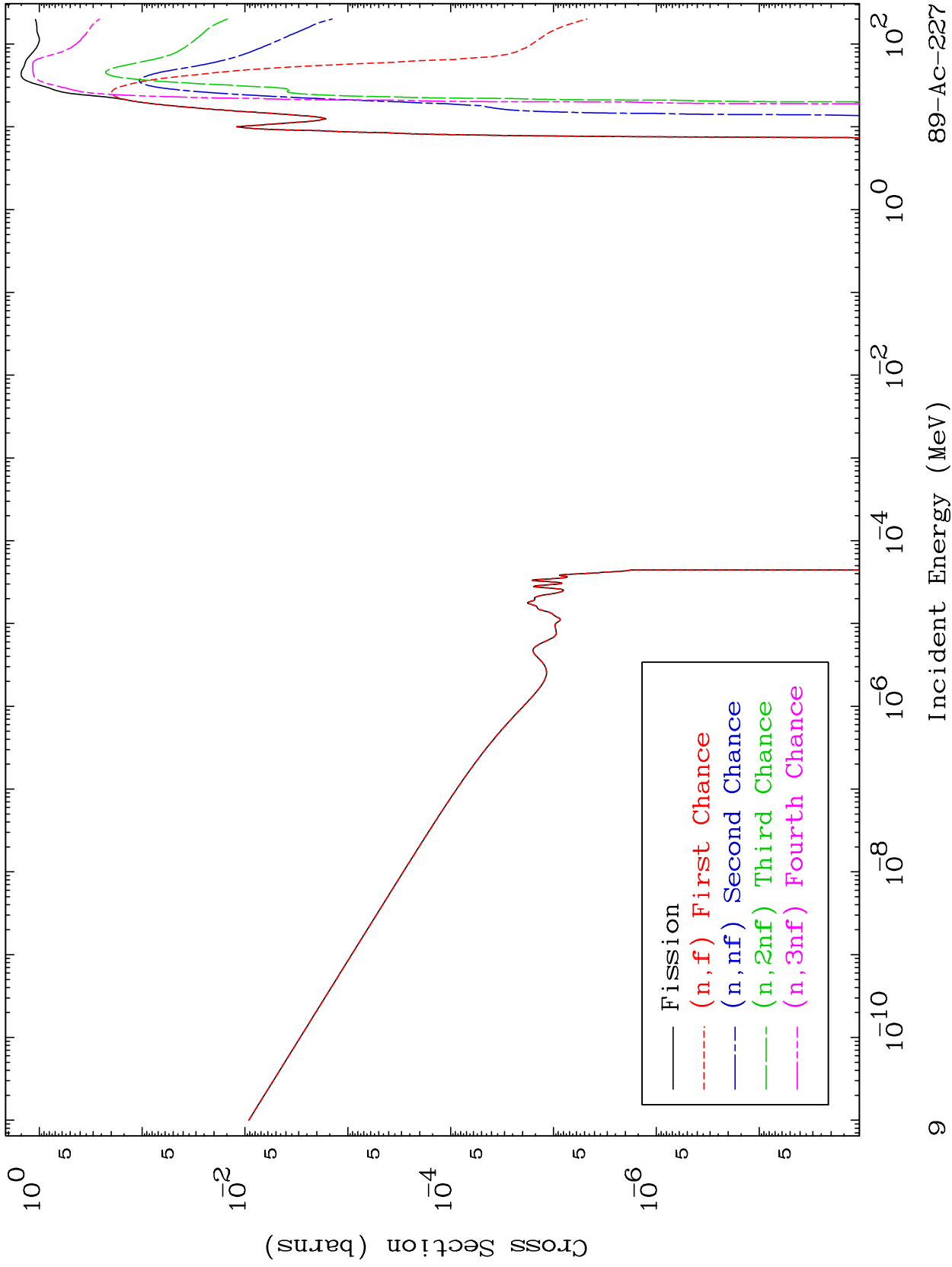


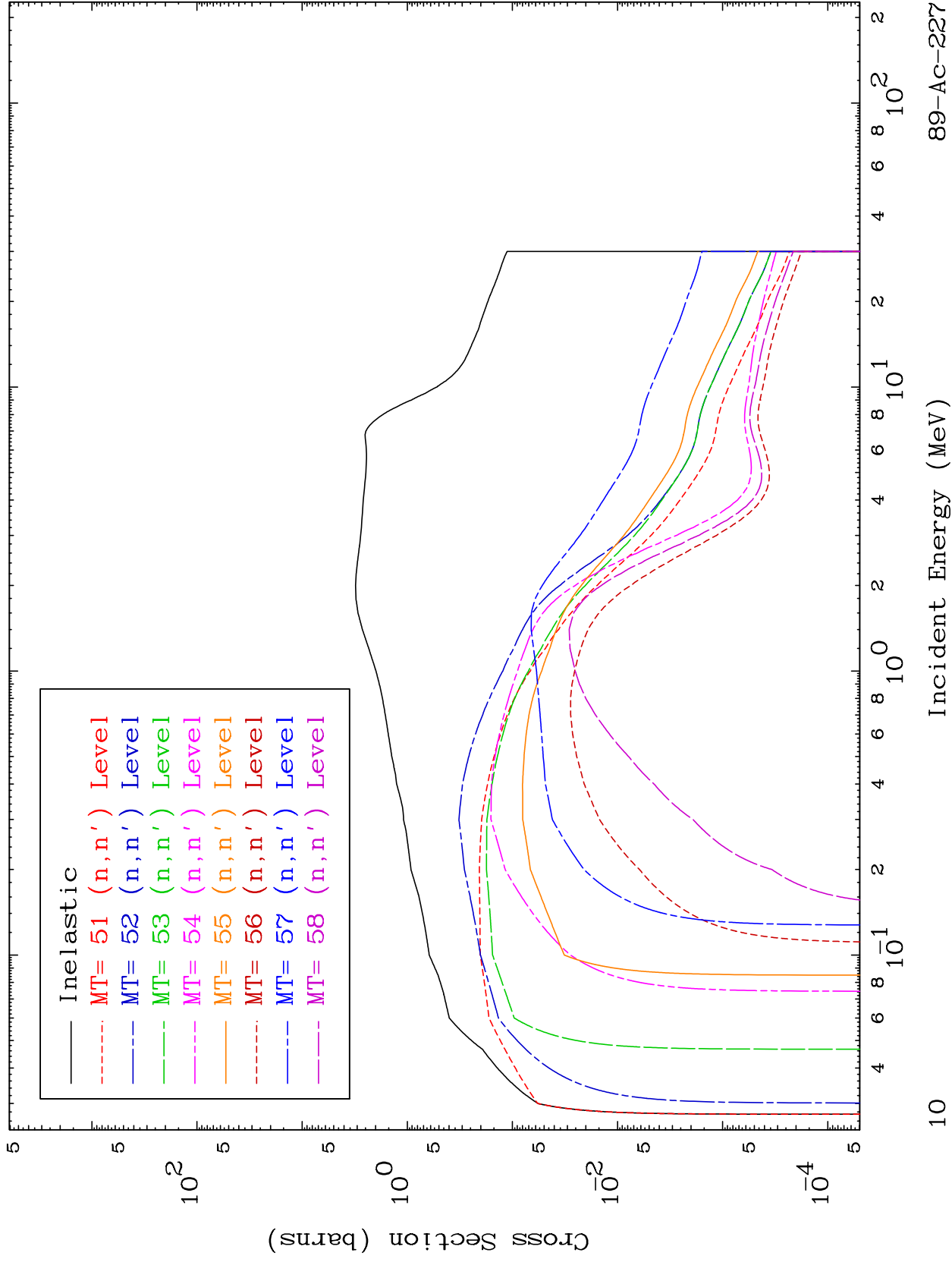


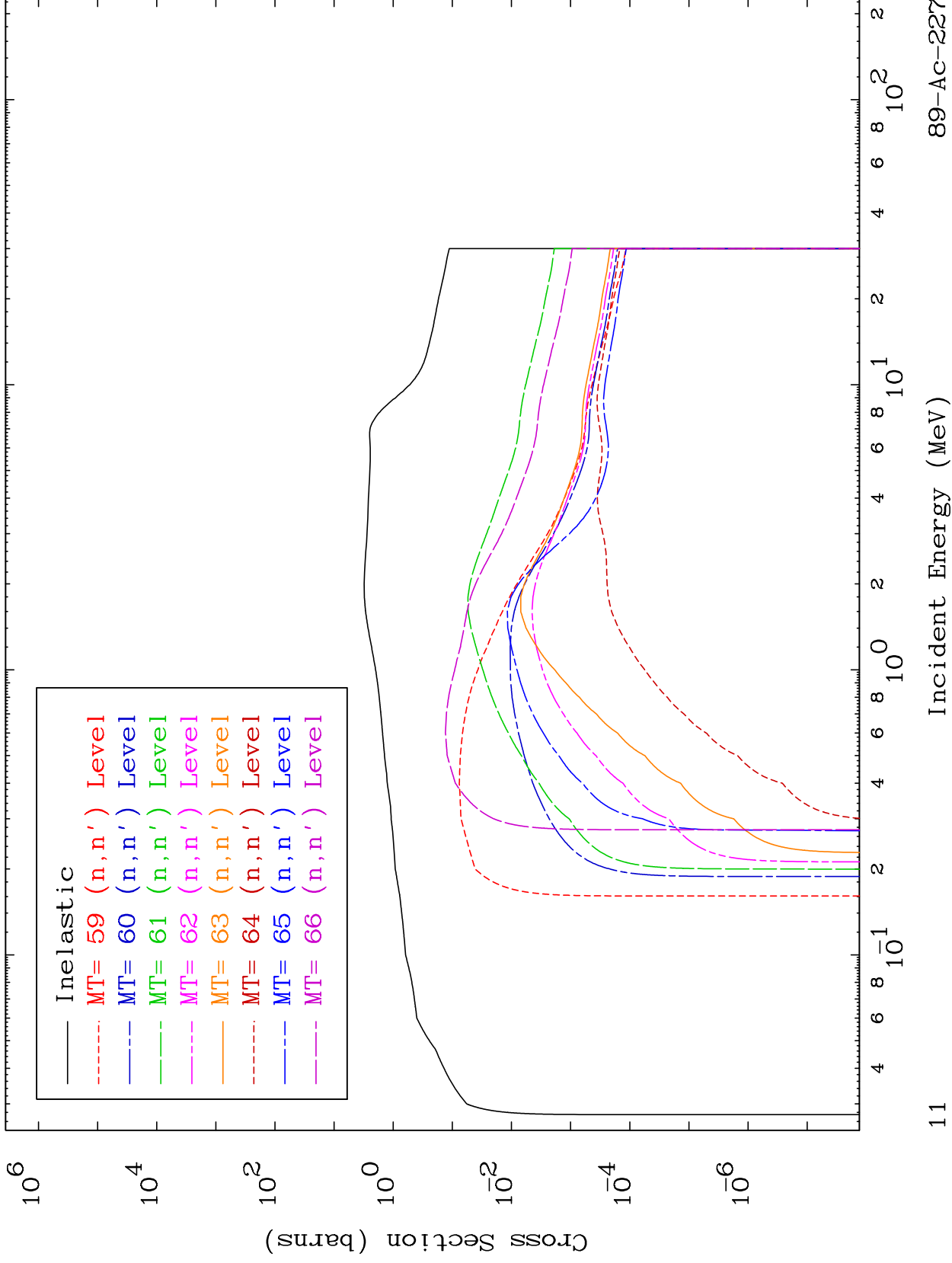
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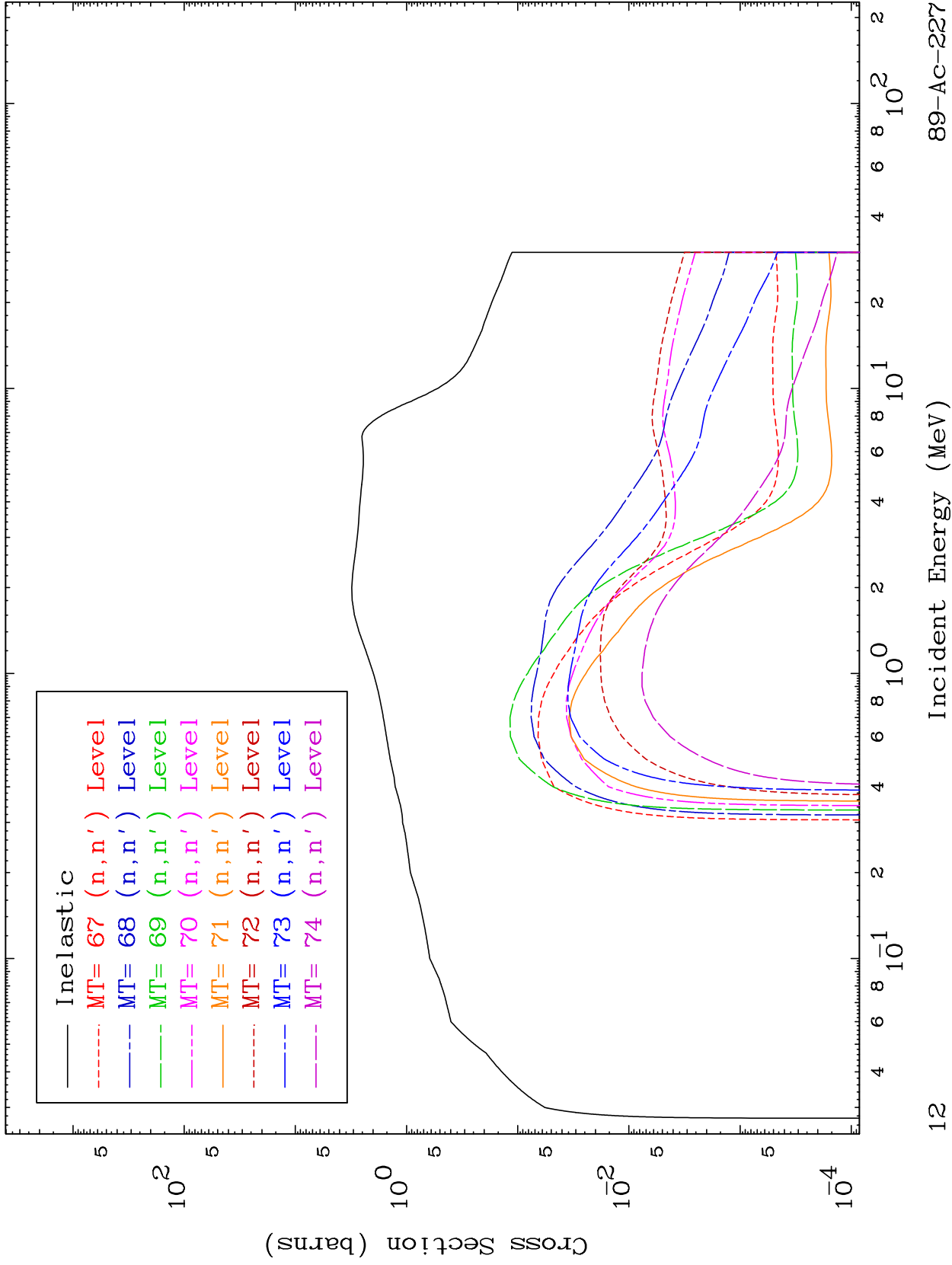
Fission
293 Kelvin Cross Sections

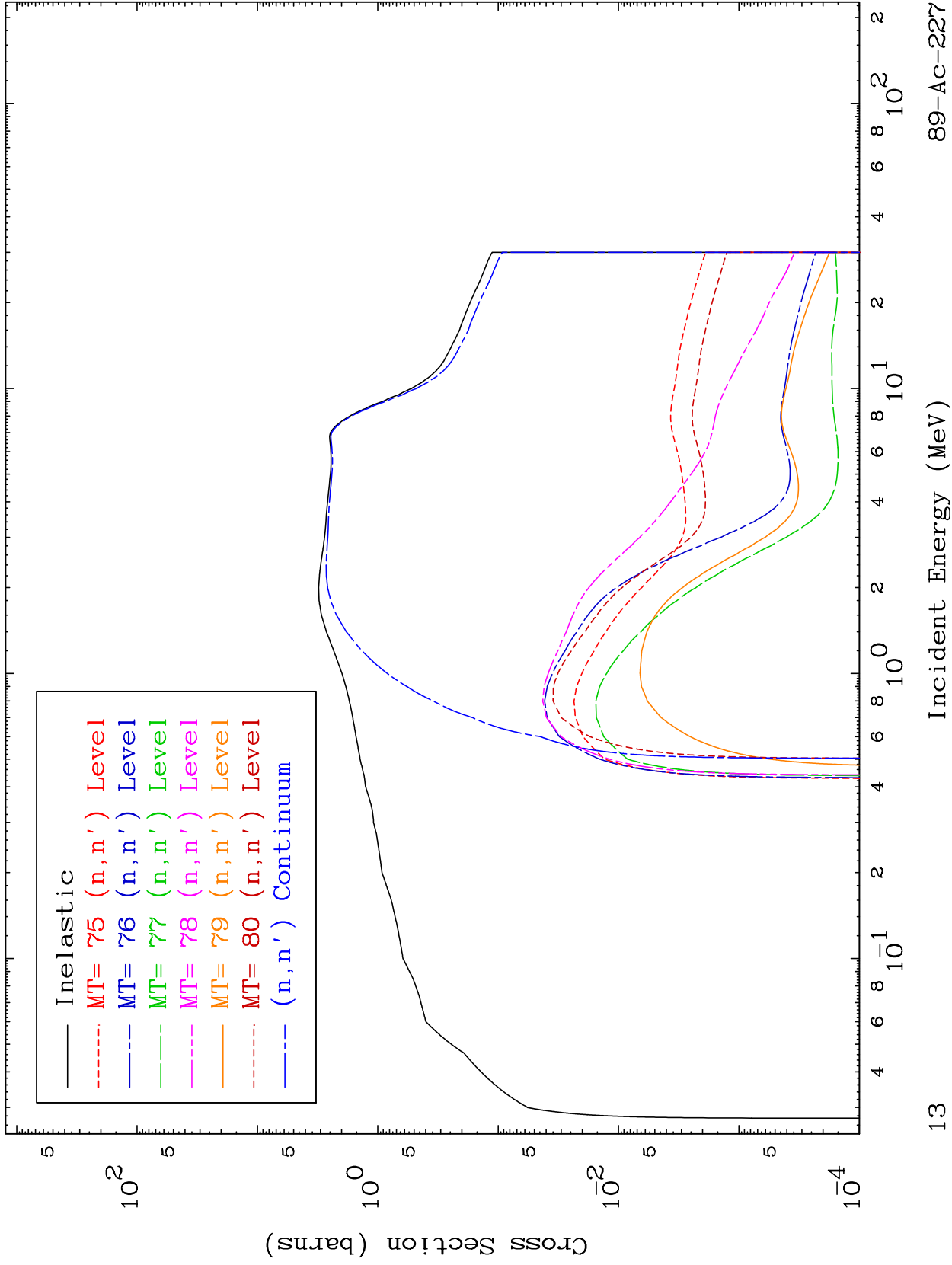
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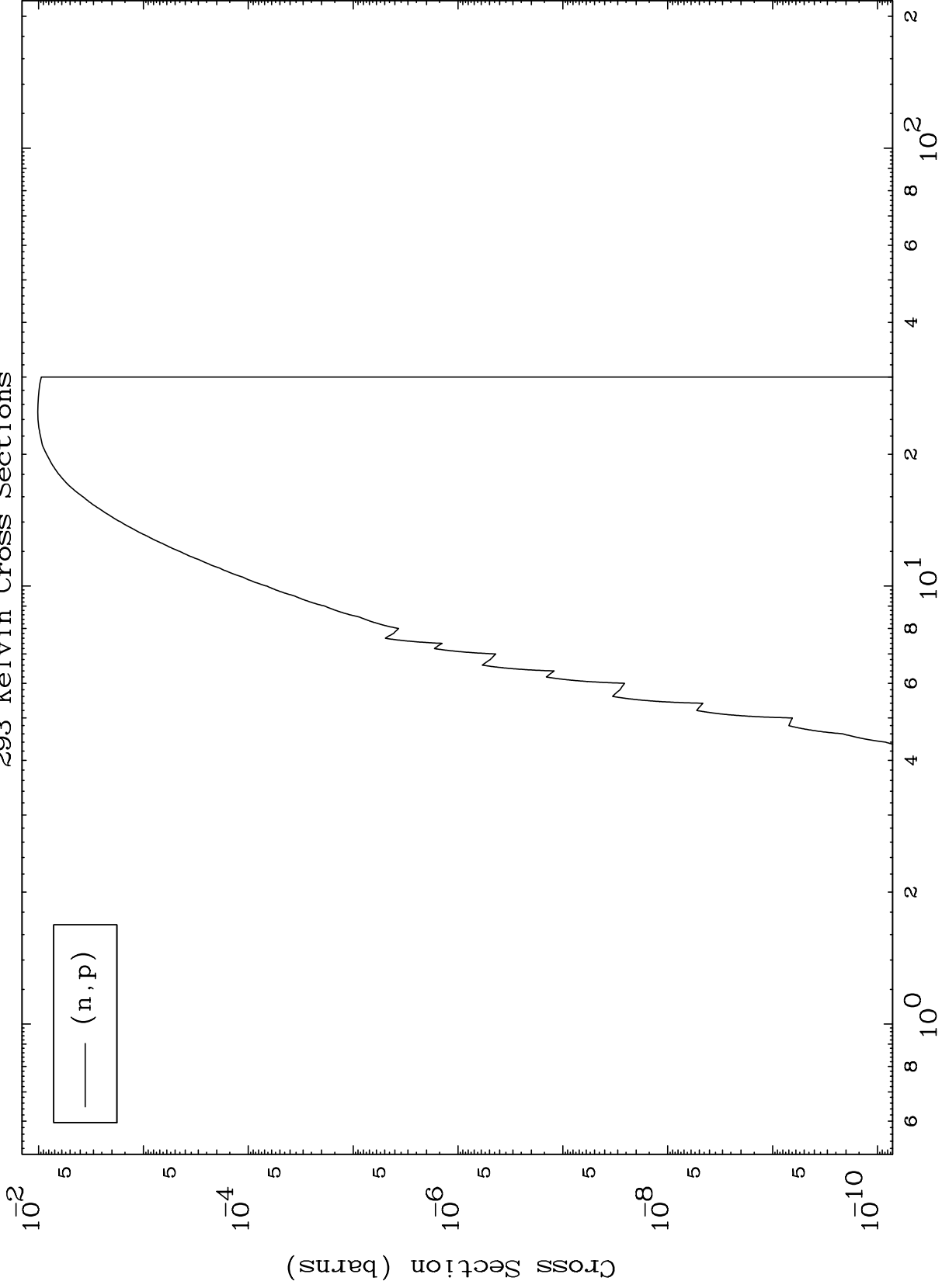


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(n,p) Levels

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293 Kelvin Cross Sections



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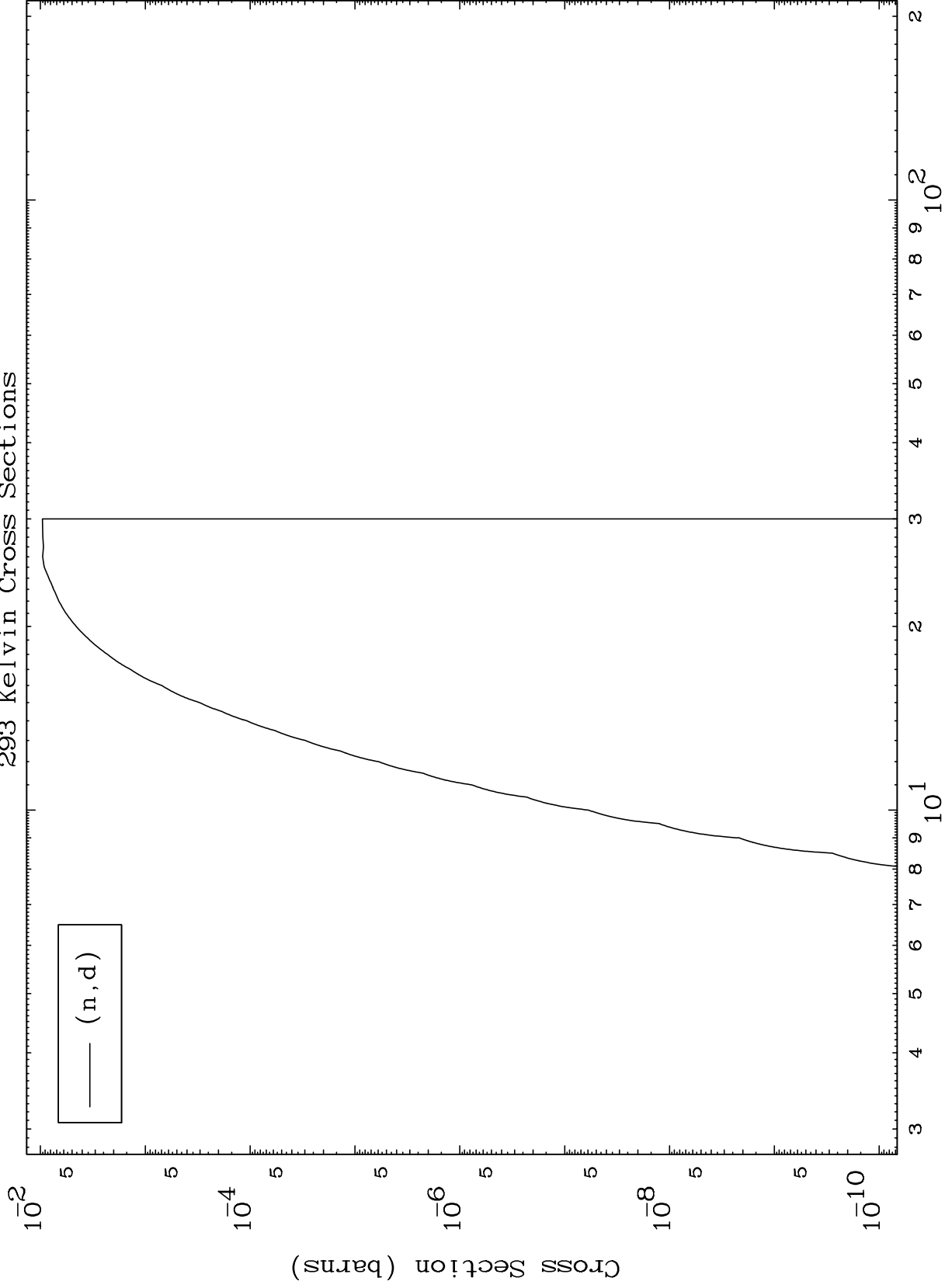
Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

89-Ac-227



15

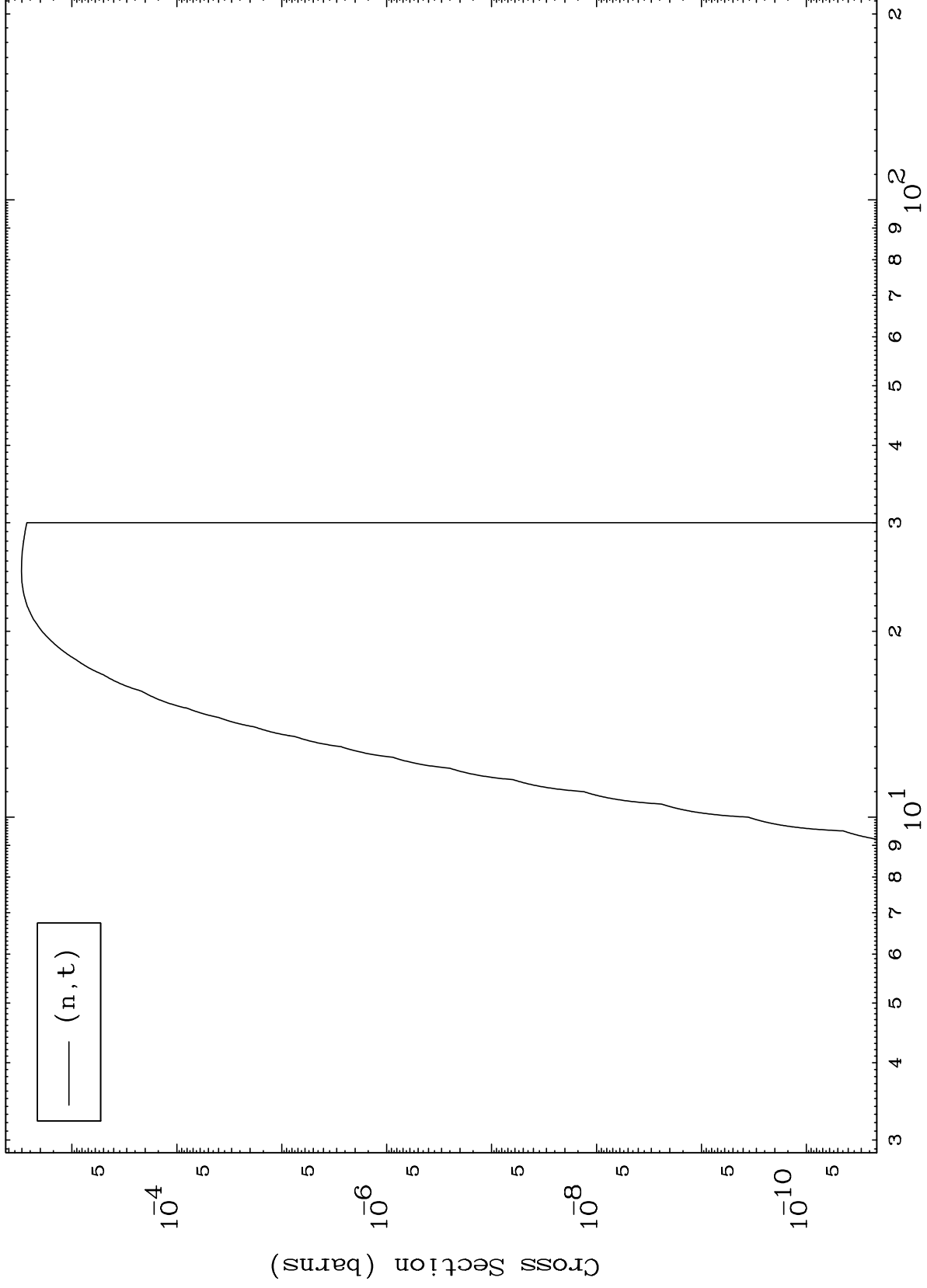
Incident Energy (MeV)

89-Ac-227

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(n,t) Levels
293 Kelvin Cross Sections

89-Ac-227



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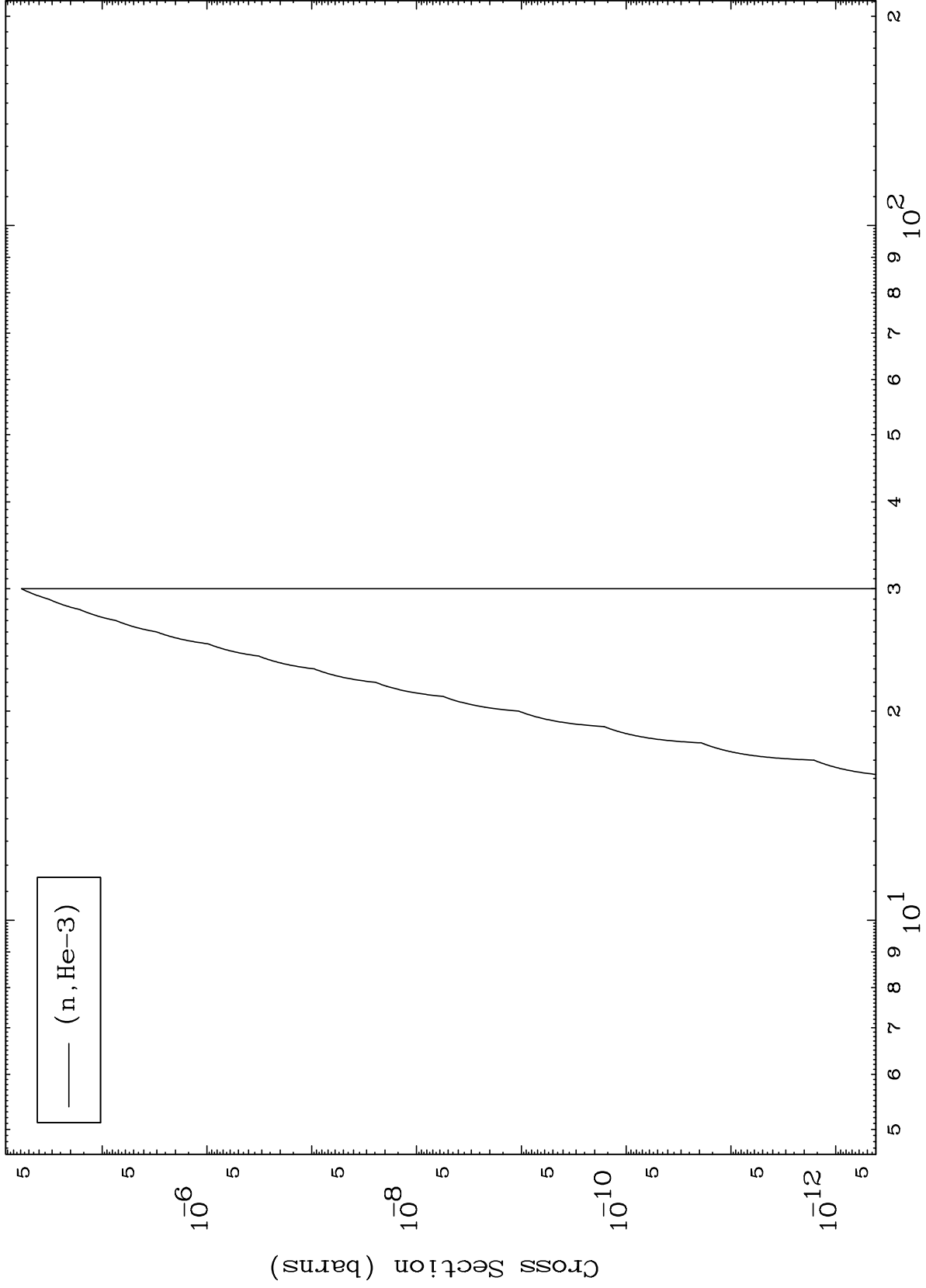
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

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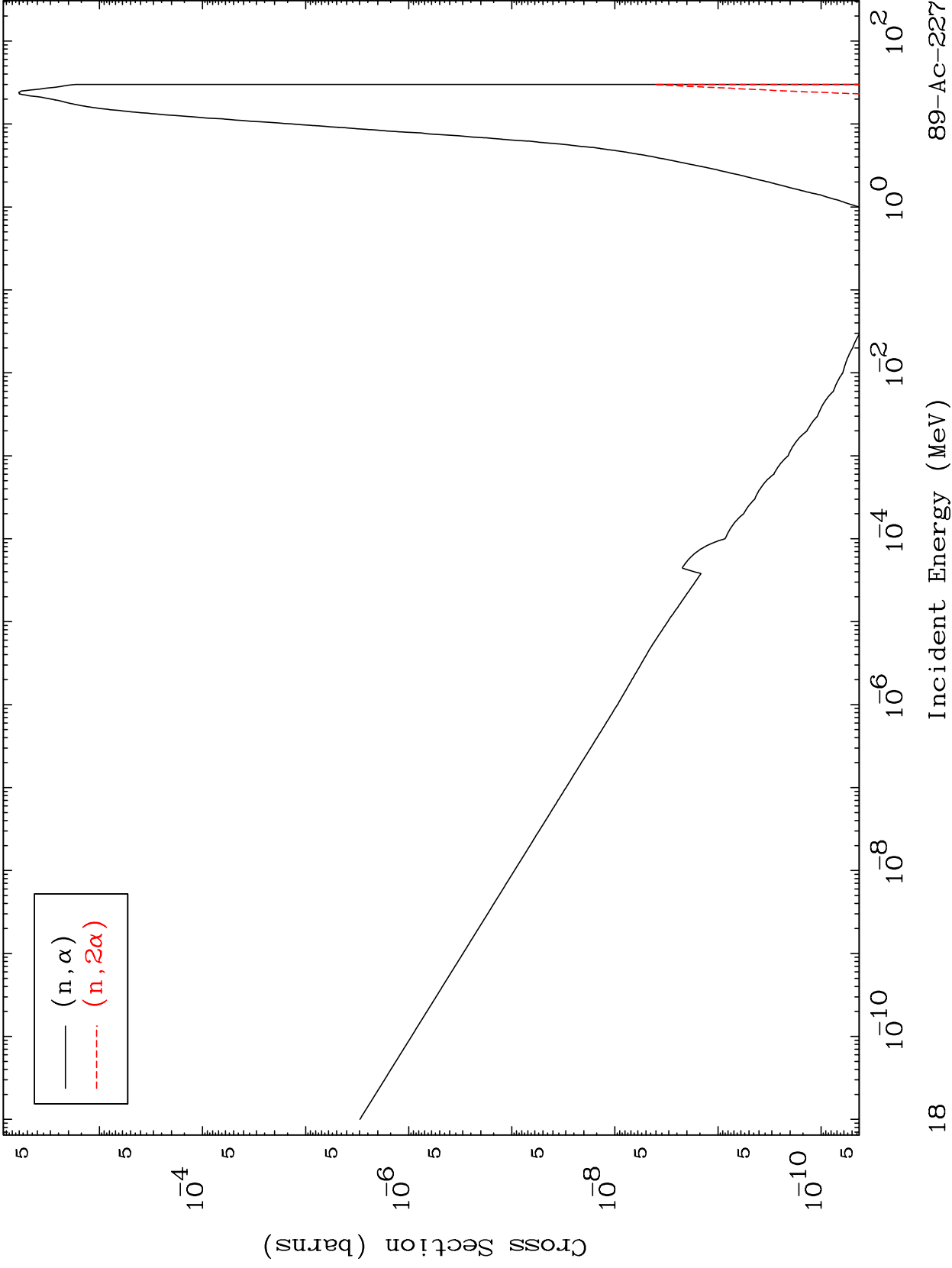
Incident Energy (MeV)

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(n, α) Levels
293 Kelvin Cross Sections

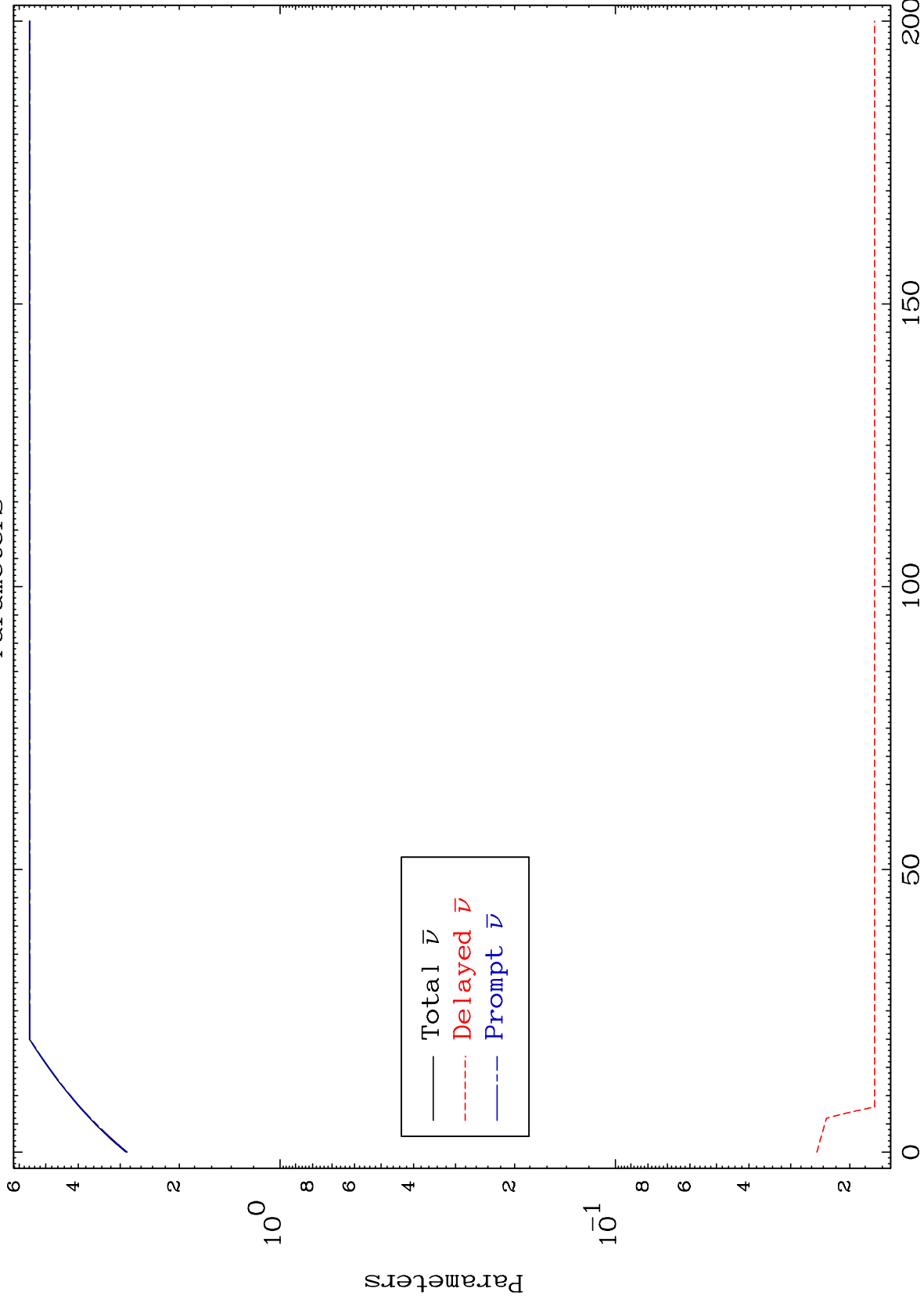
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Energy Release
Parameters

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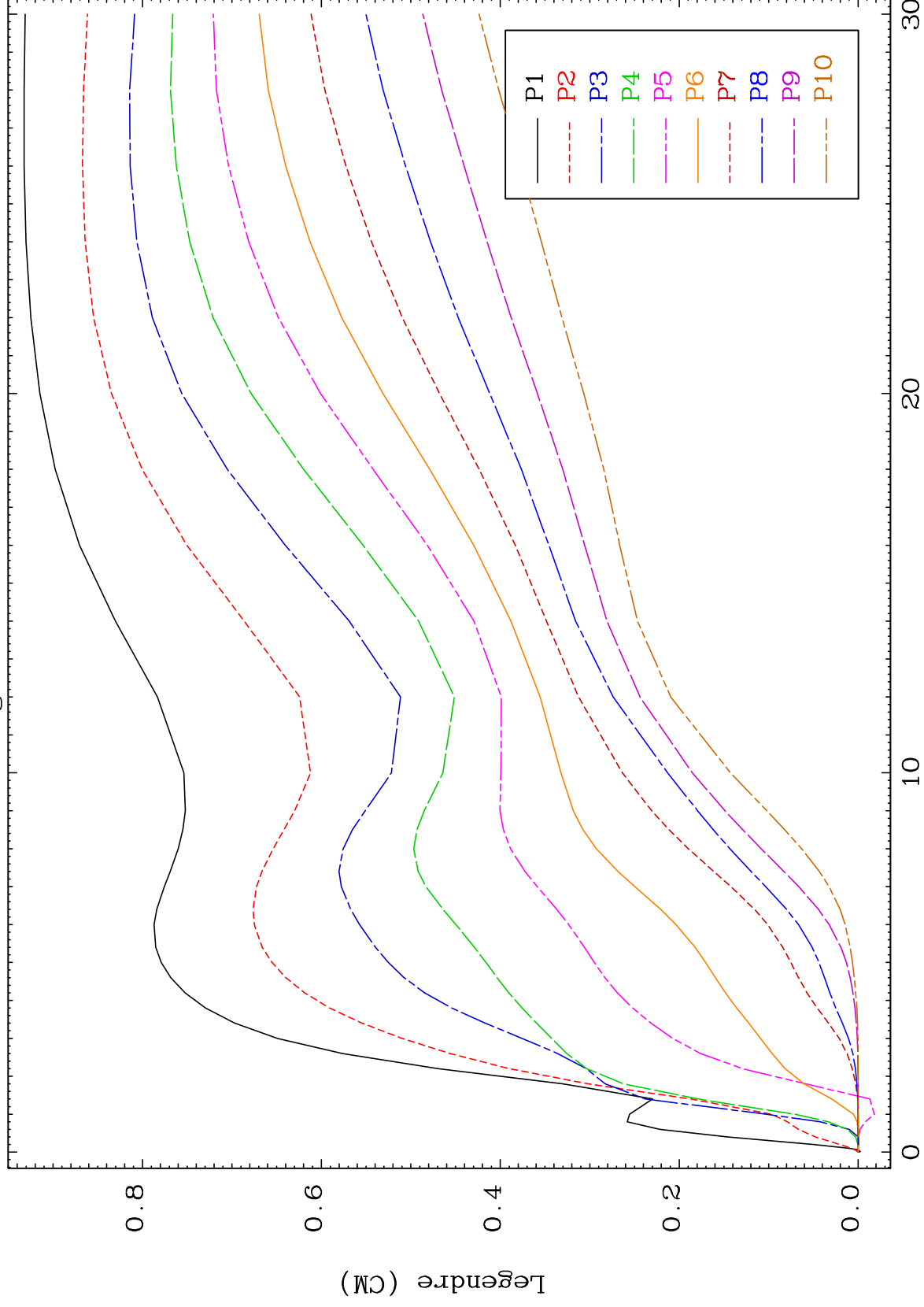
Incident Energy (MeV)

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Elastic Legendre Coefficients

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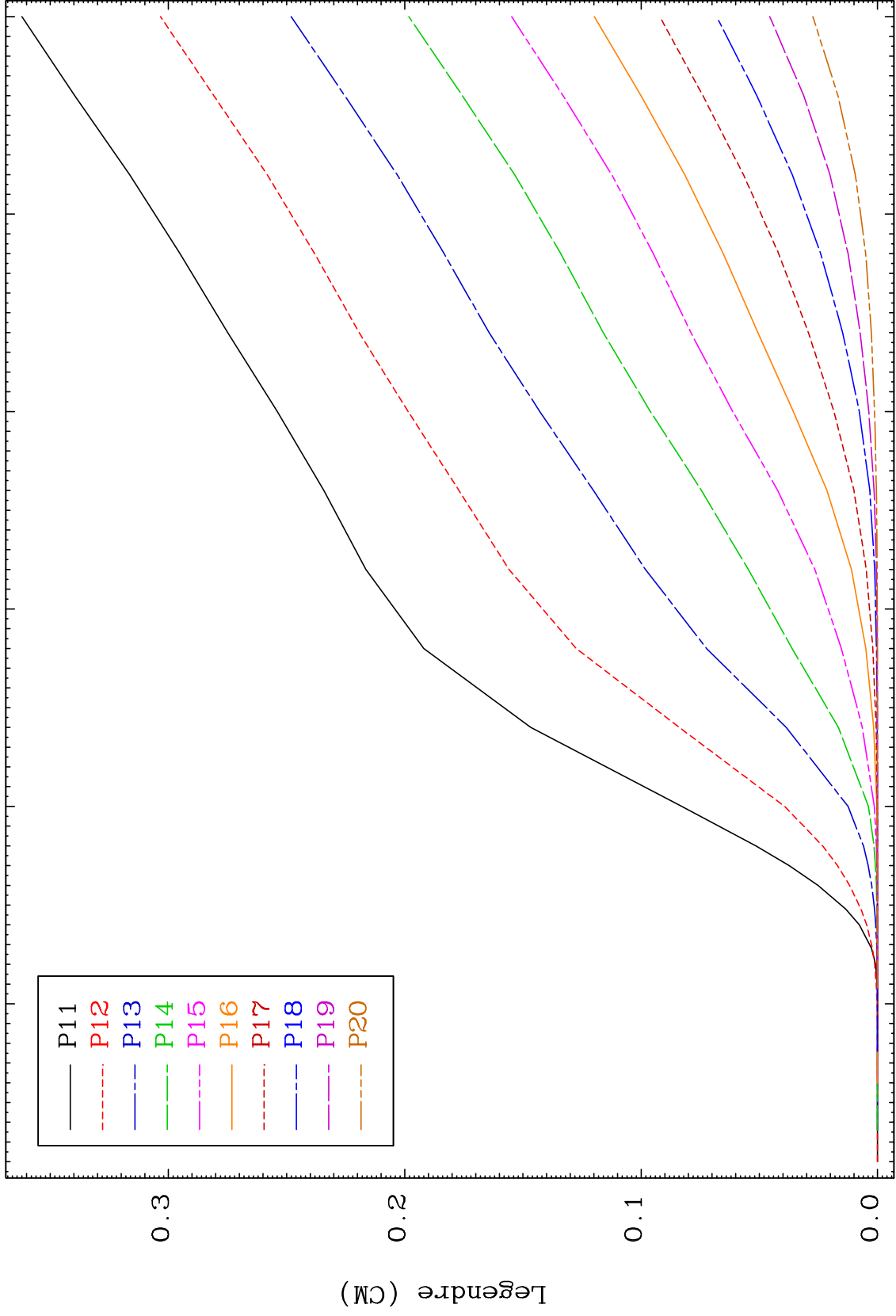
Incident Energy (MeV)

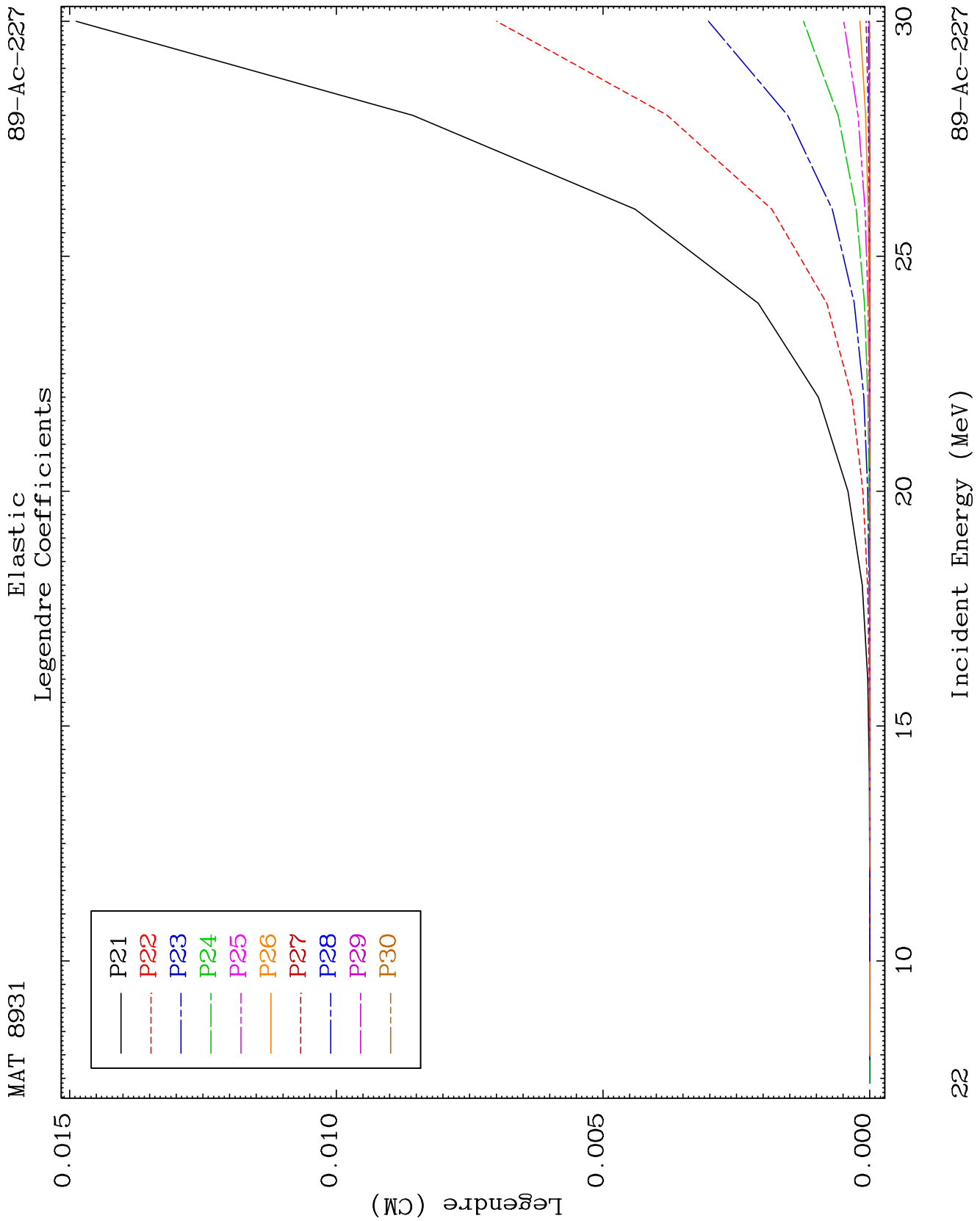
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20

30

Elastic Legendre Coefficients

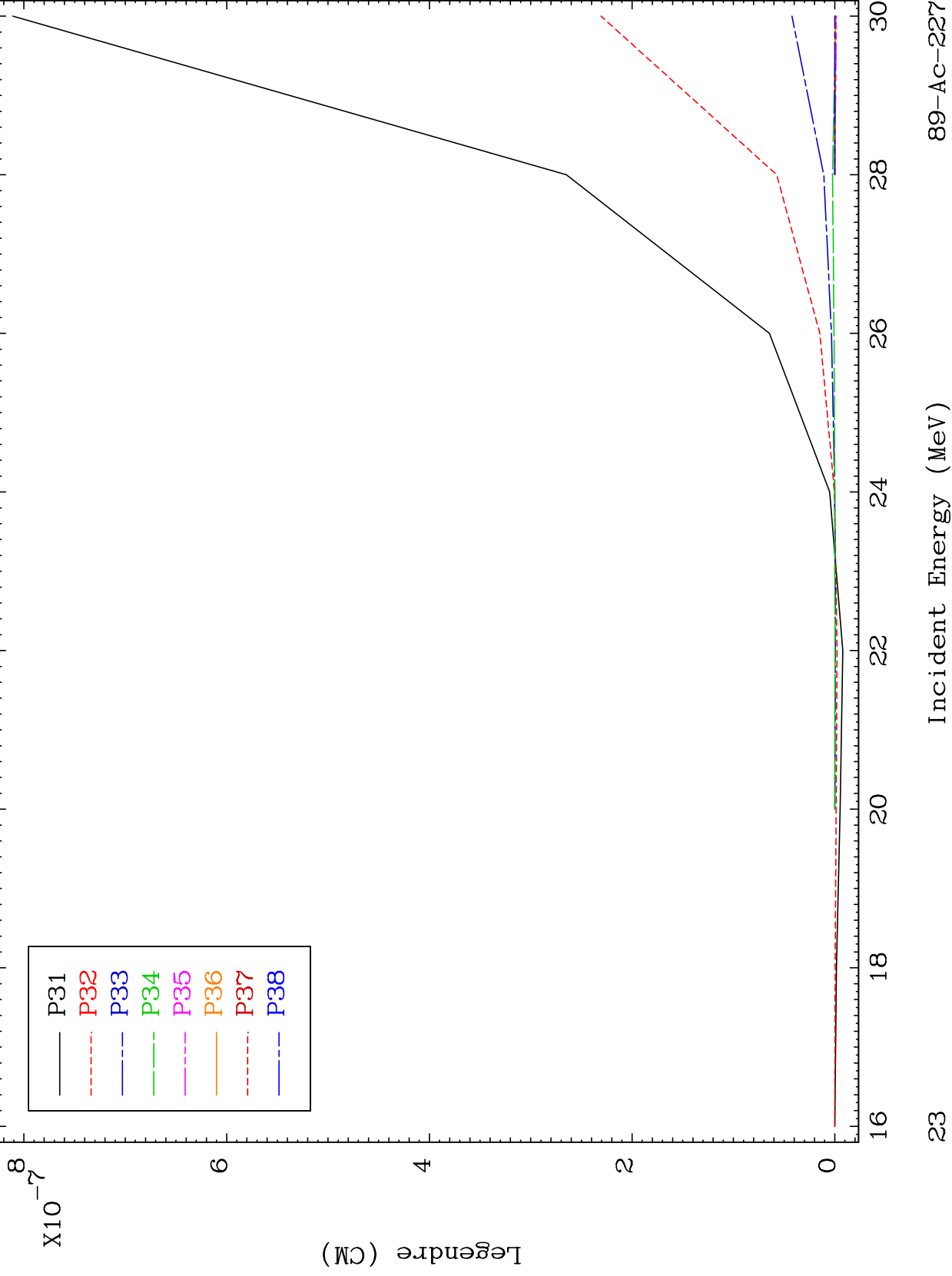




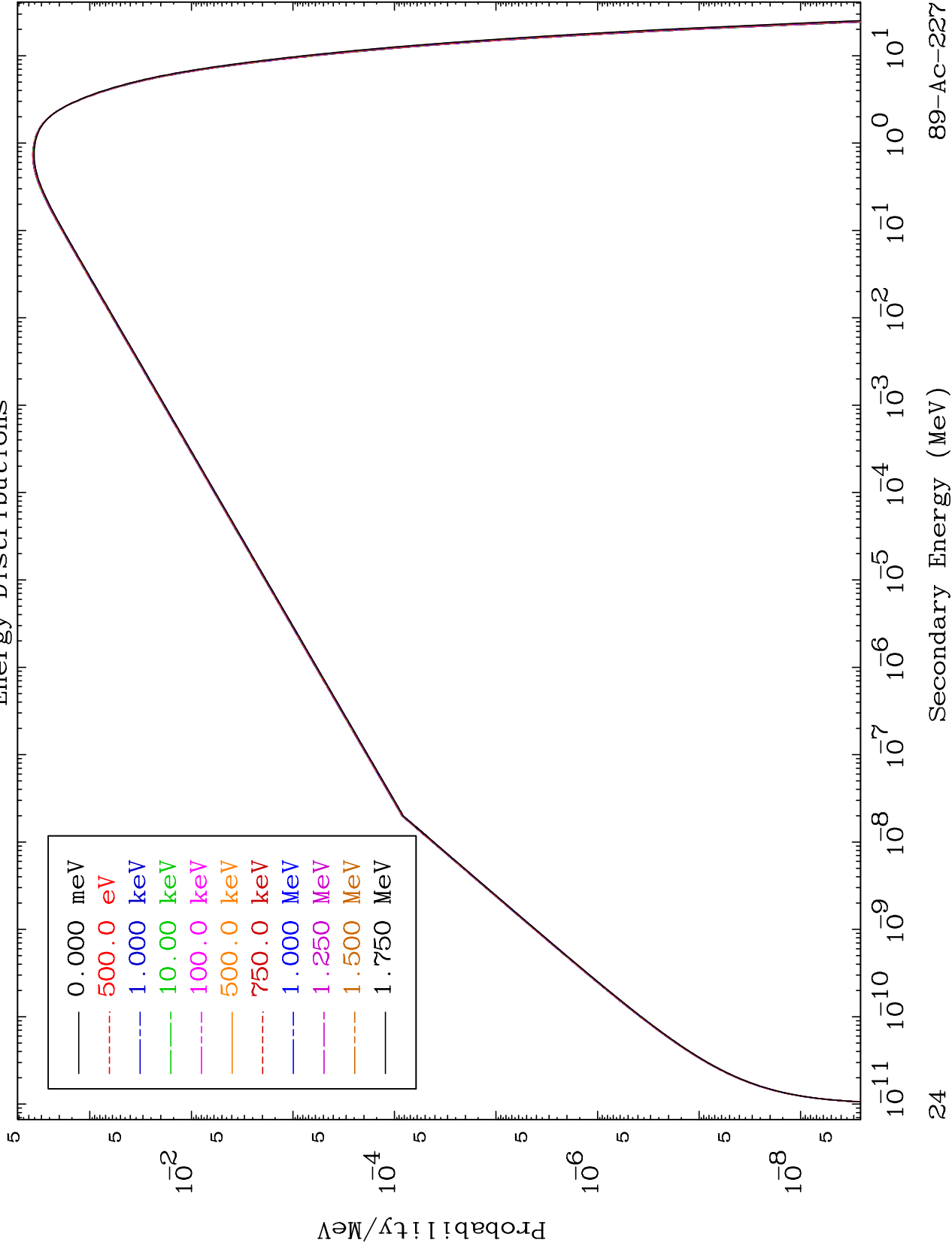
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Elastic Legendre Coefficients

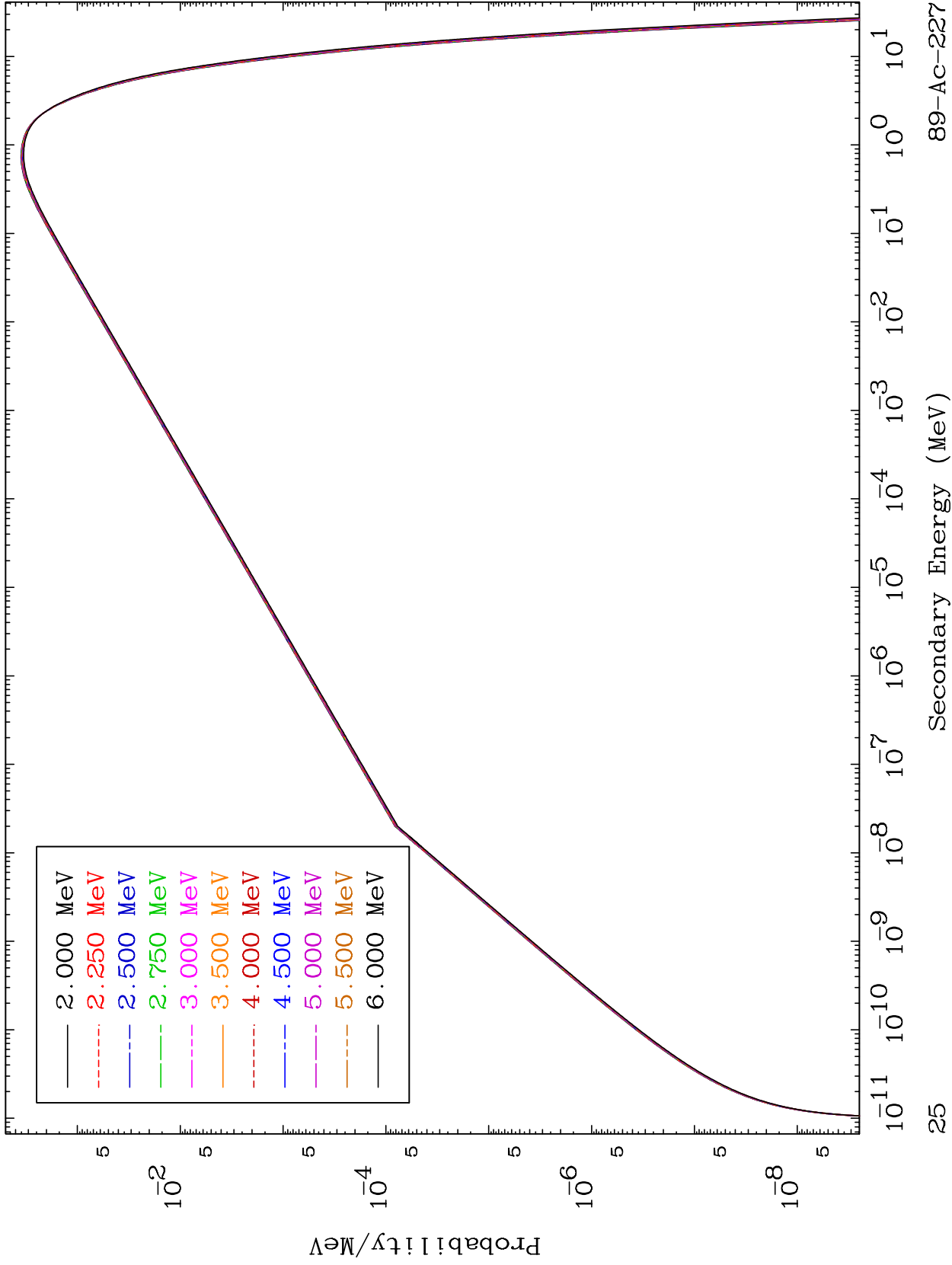
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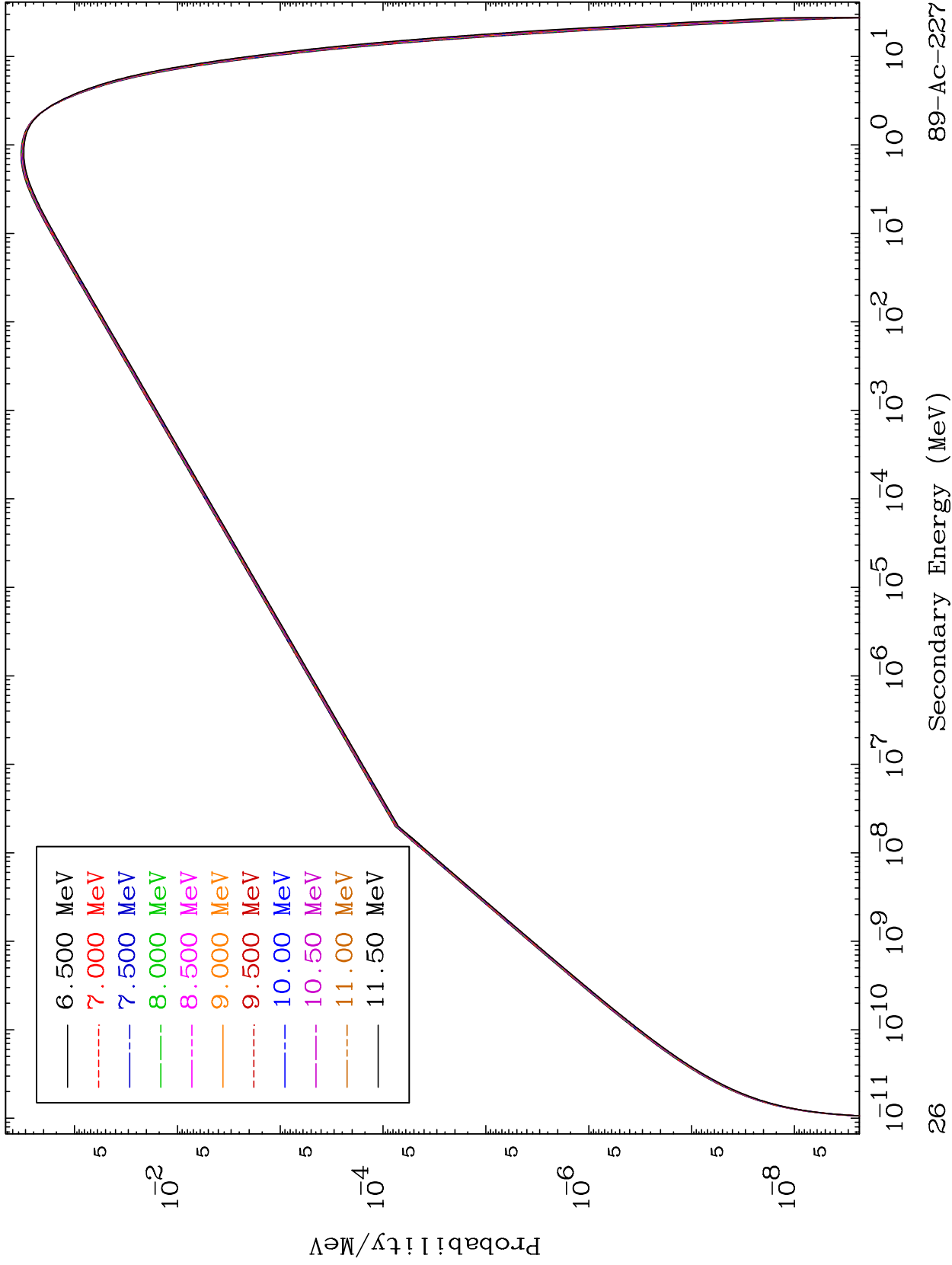


Fission Energy Distributions

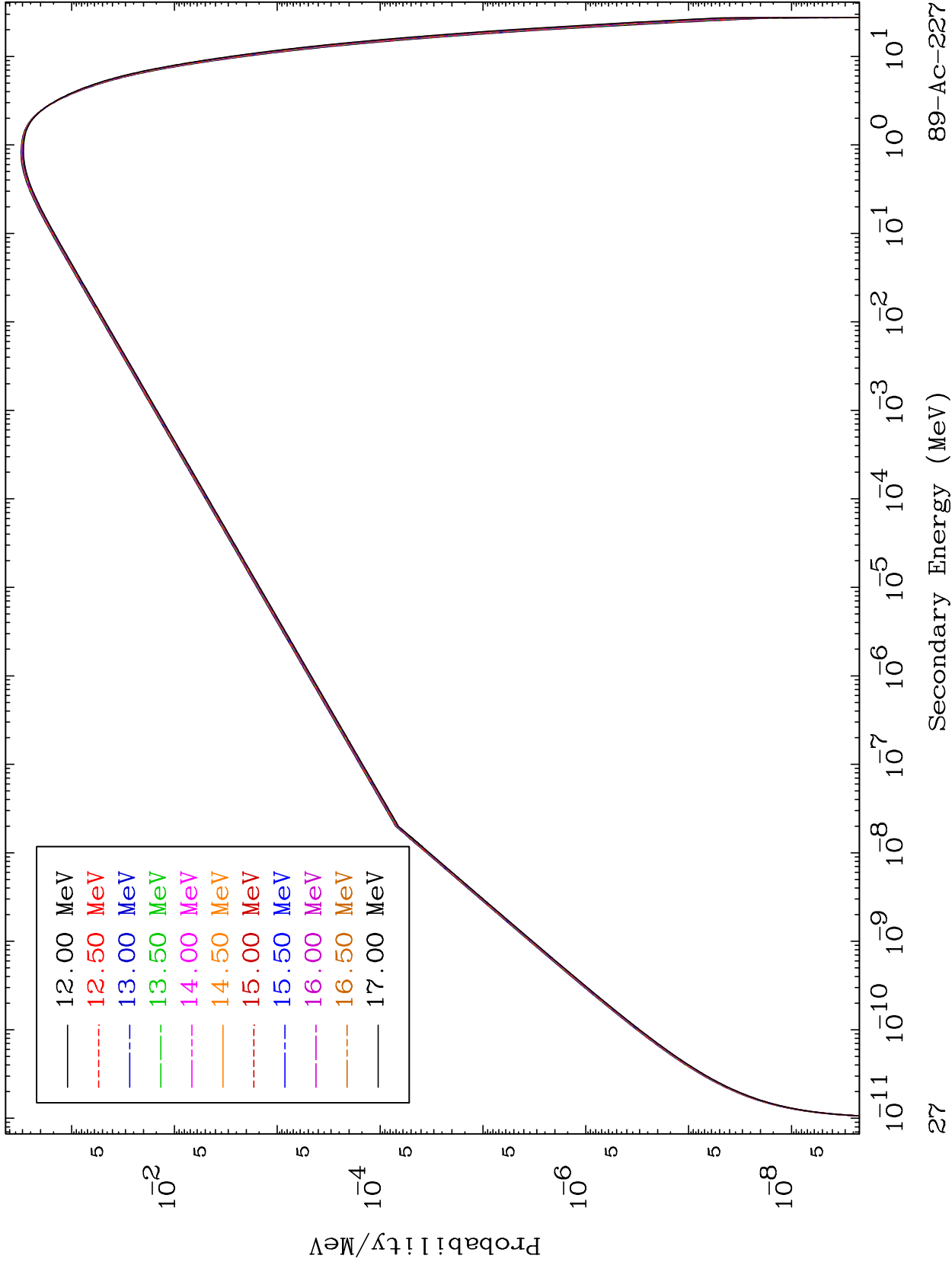


Fission Energy Distributions

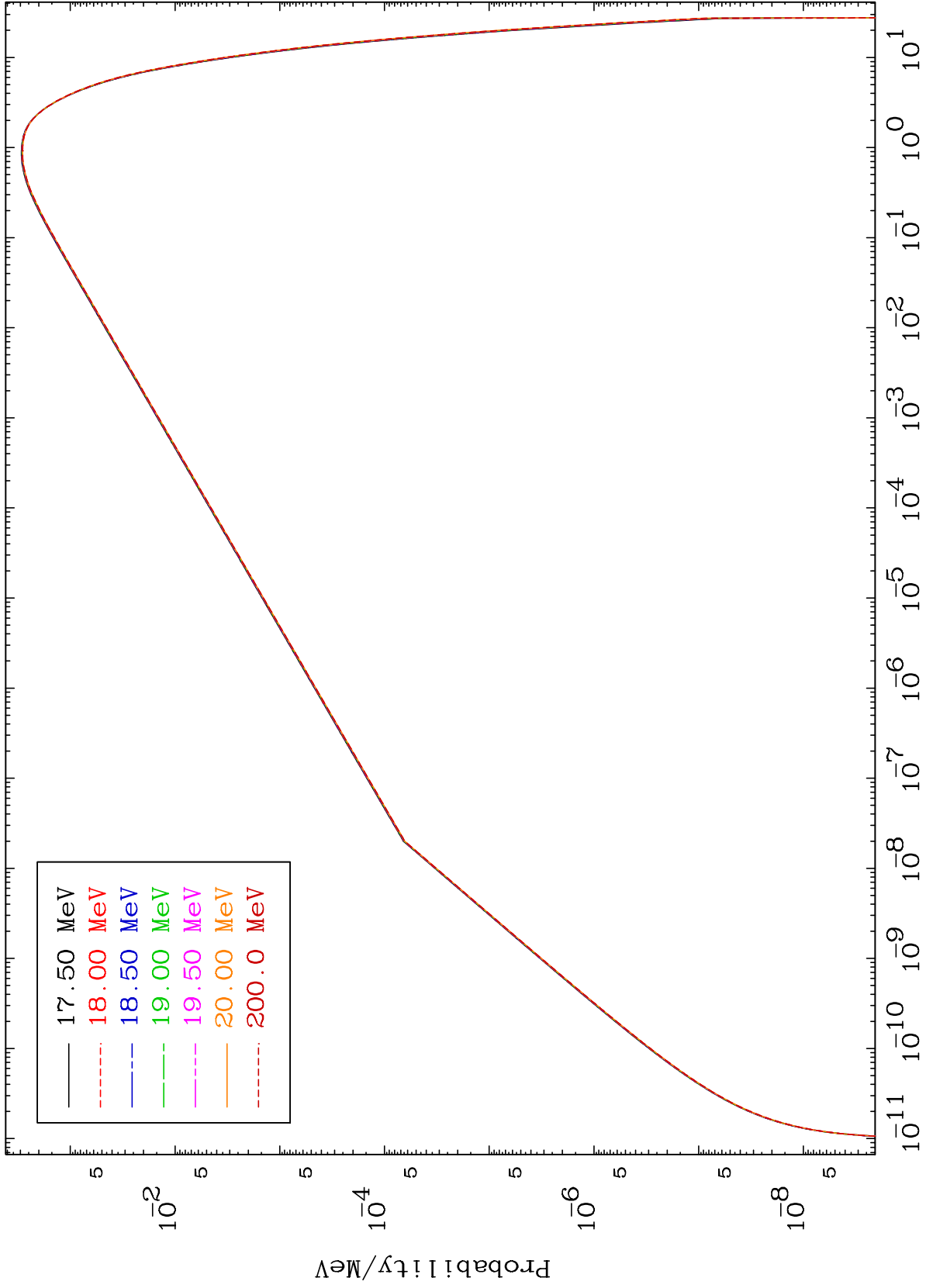




Fission Energy Distributions



Fission Energy Distributions

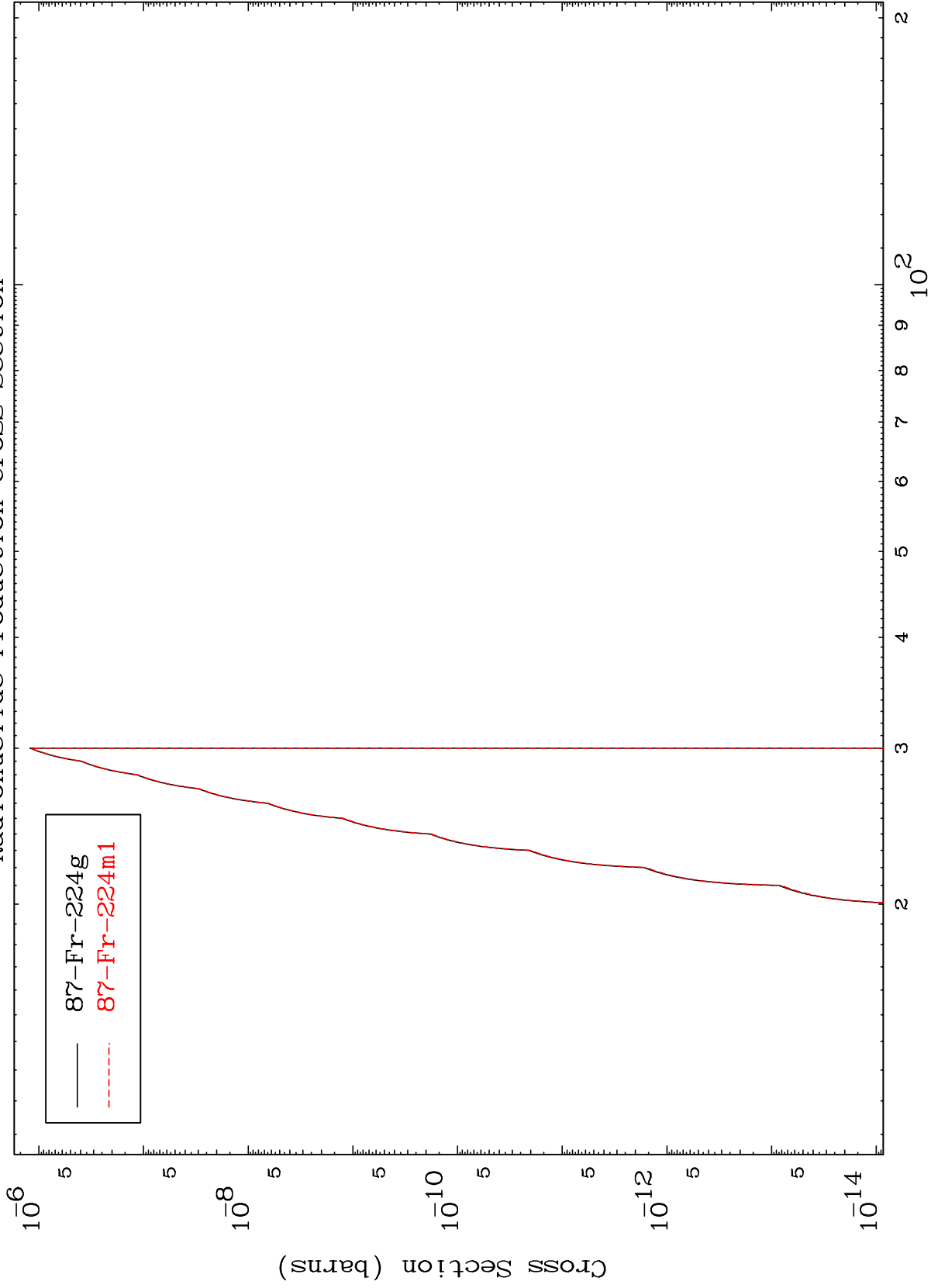


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(n,n') He-3

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Radionuclide Production Cross Section



Incident Energy (MeV)

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(n, α)

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Radionuclide Production Cross Section

