

Program EVALPLOT  
(Version 2021-1)

by

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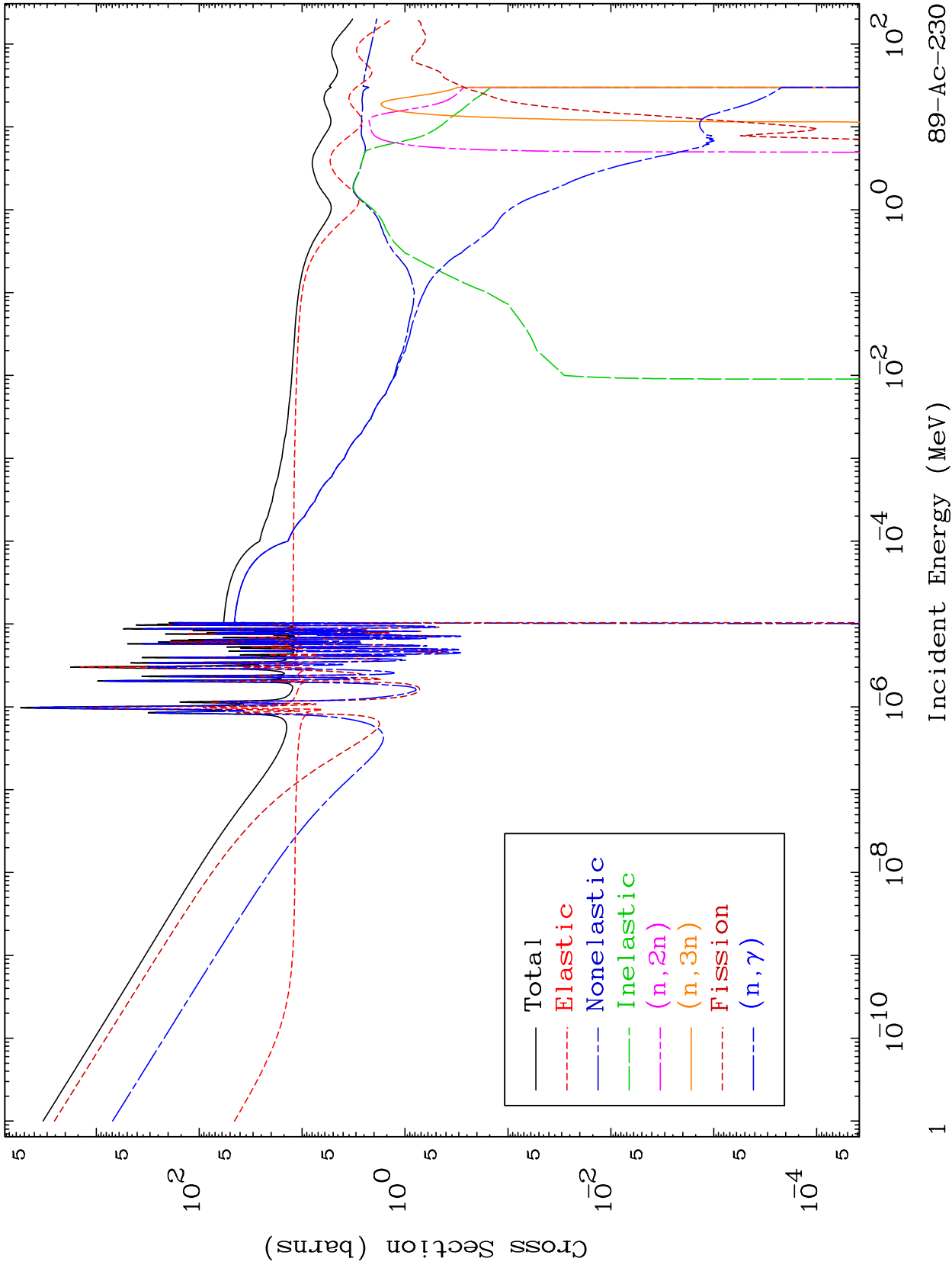
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8940

Neutron Major  
293 Kelvin Cross Sections

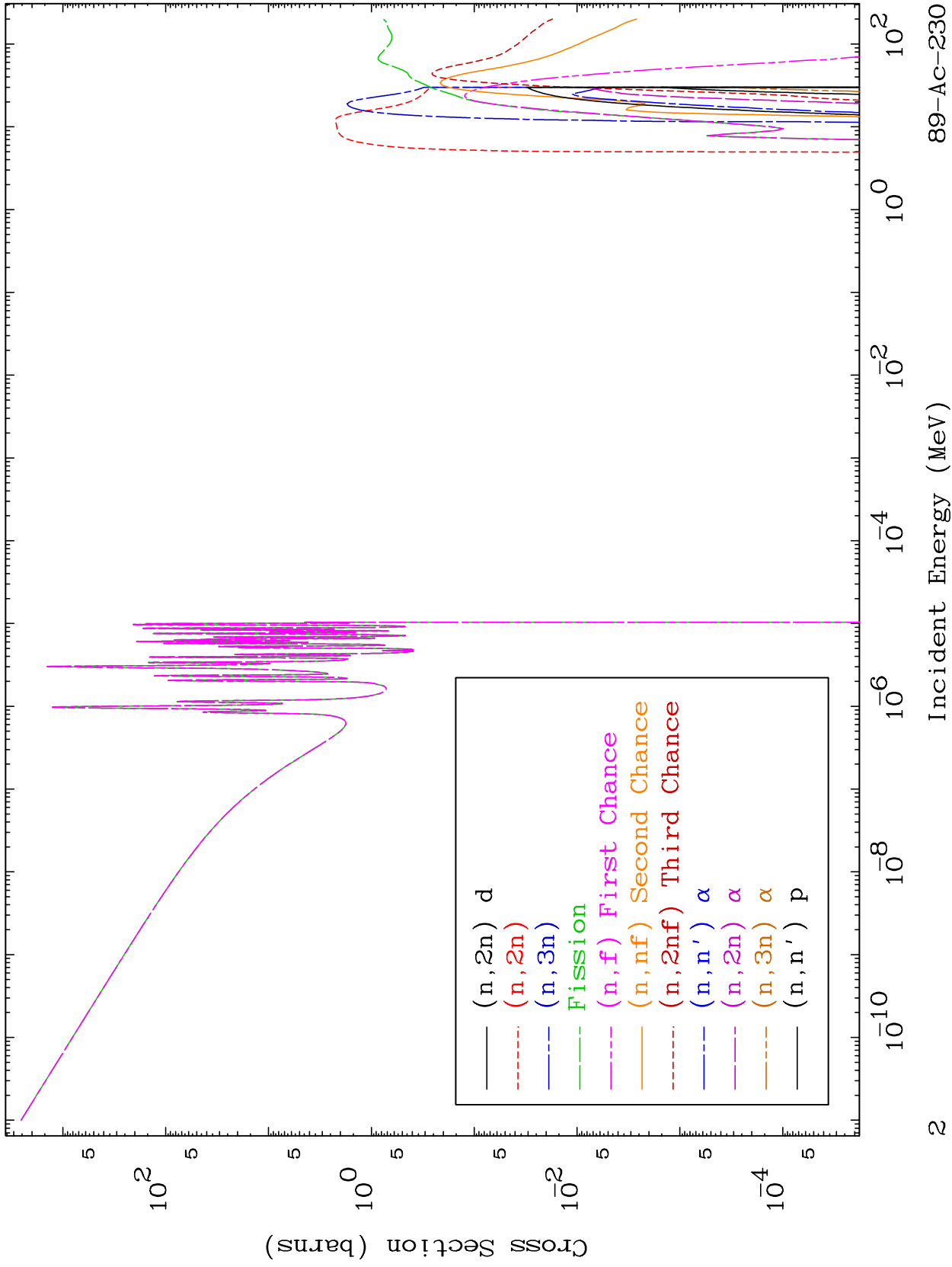
89-Ac-230



MAT 8940

Neutron Absorption  
293 Kelvin Cross Sections

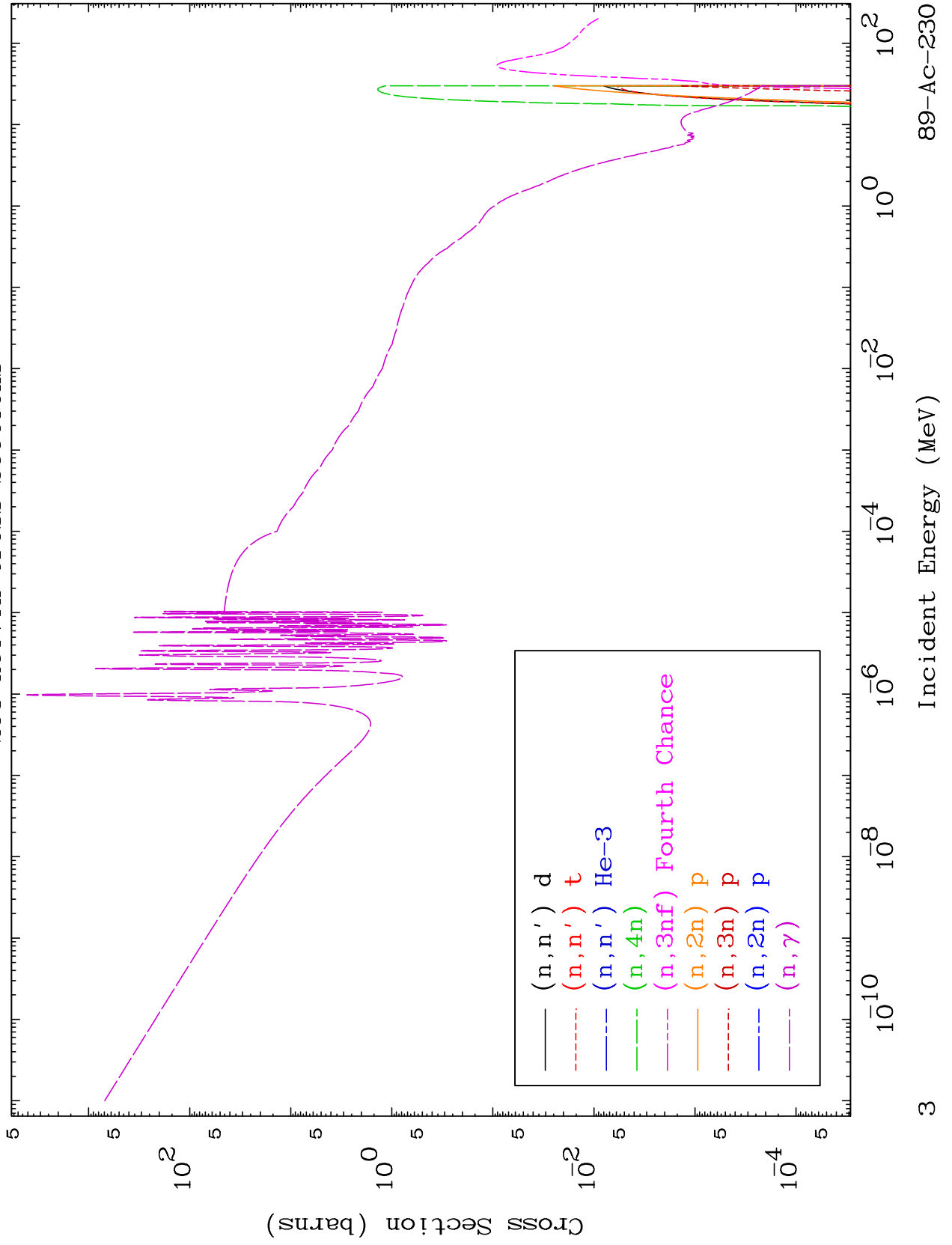
89-Ac-230



MAT 8940

Neutron Absorption  
293 Kelvin Cross Sections

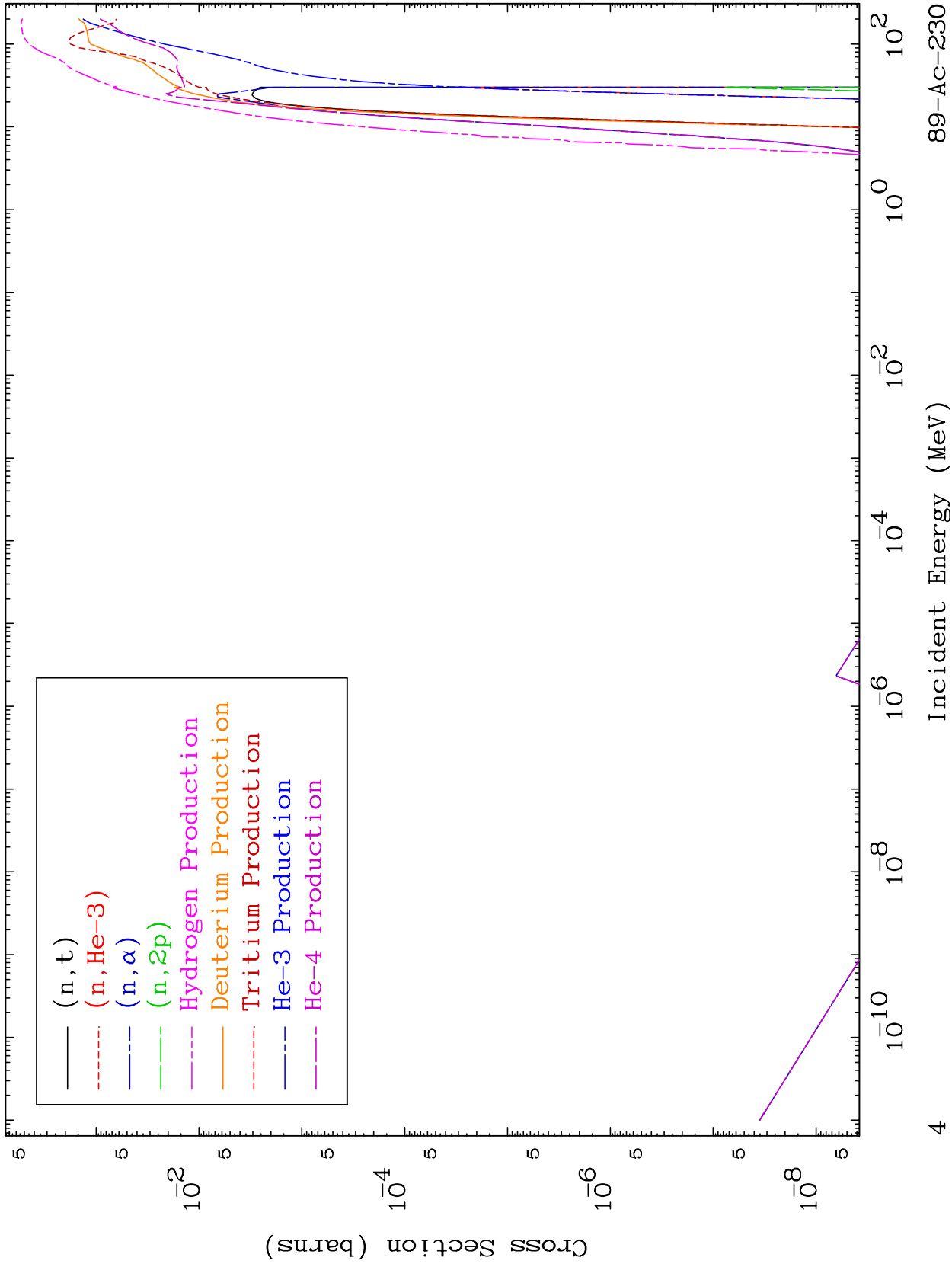
89-Ac-230



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Neutron Absorption  
293 Kelvin Cross Sections

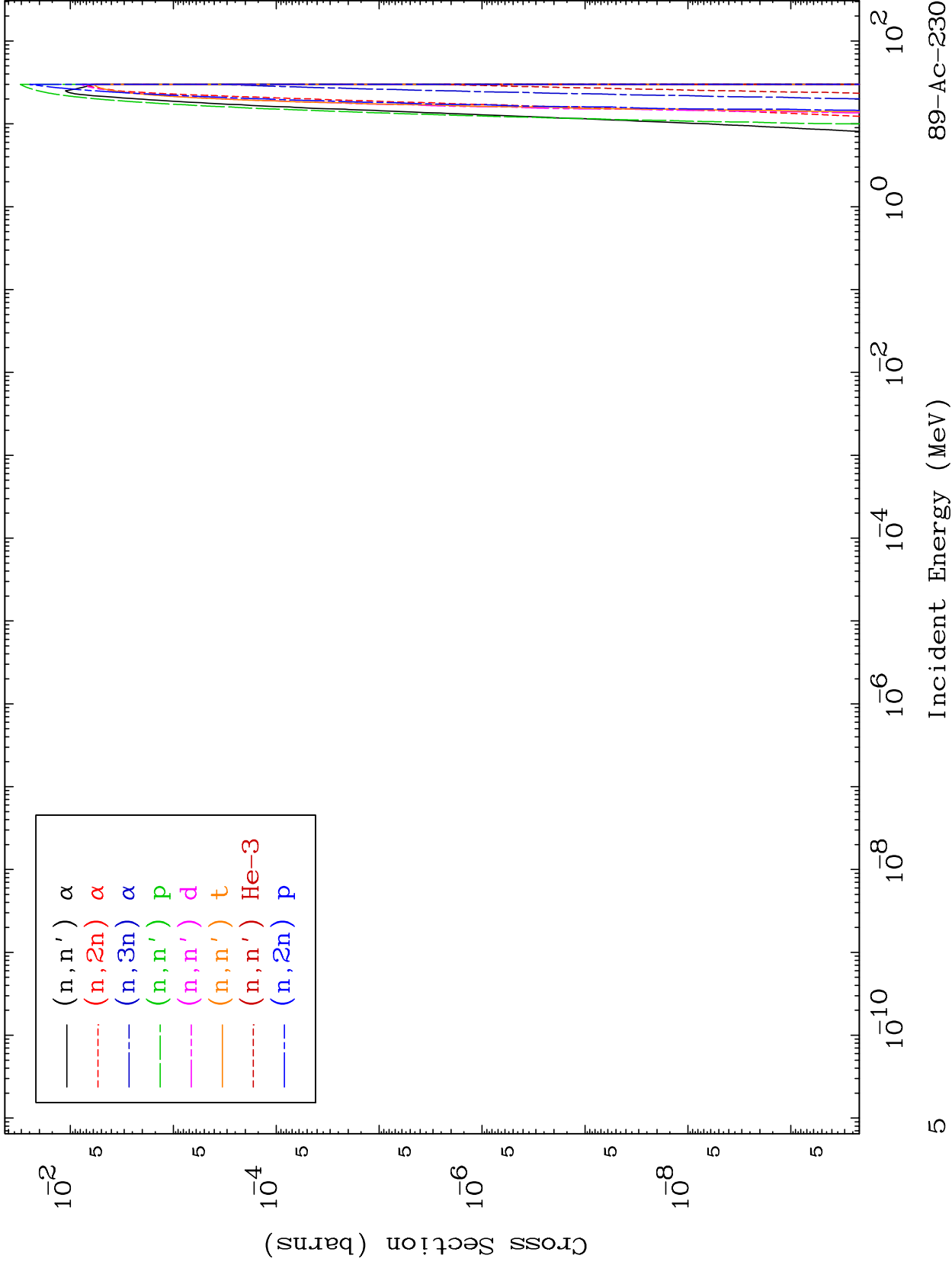
89-Ac-230



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Charged Particle  
293 Kelvin Cross Sections

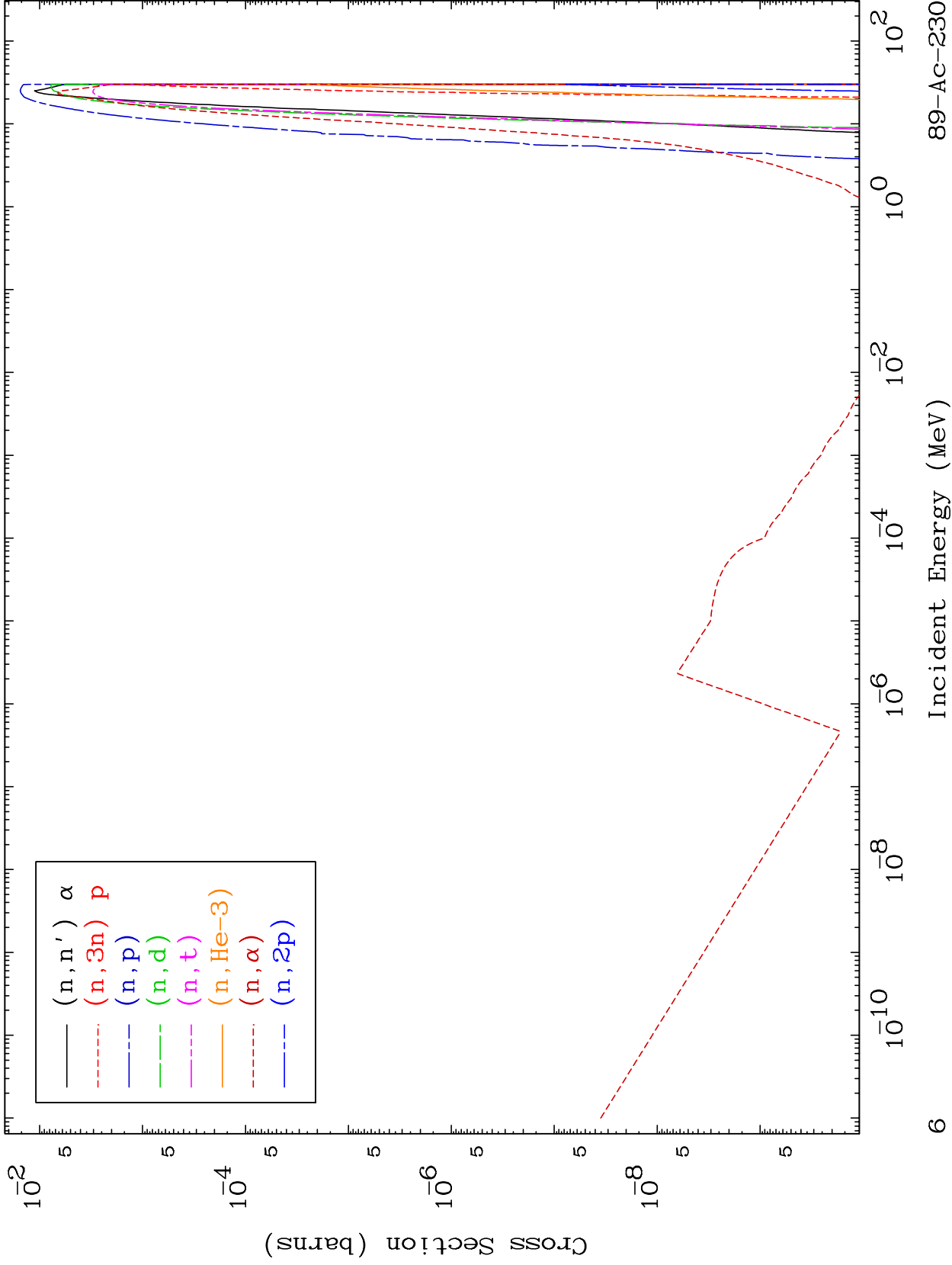
89-Ac-230



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Charged Particle  
293 Kelvin Cross Sections

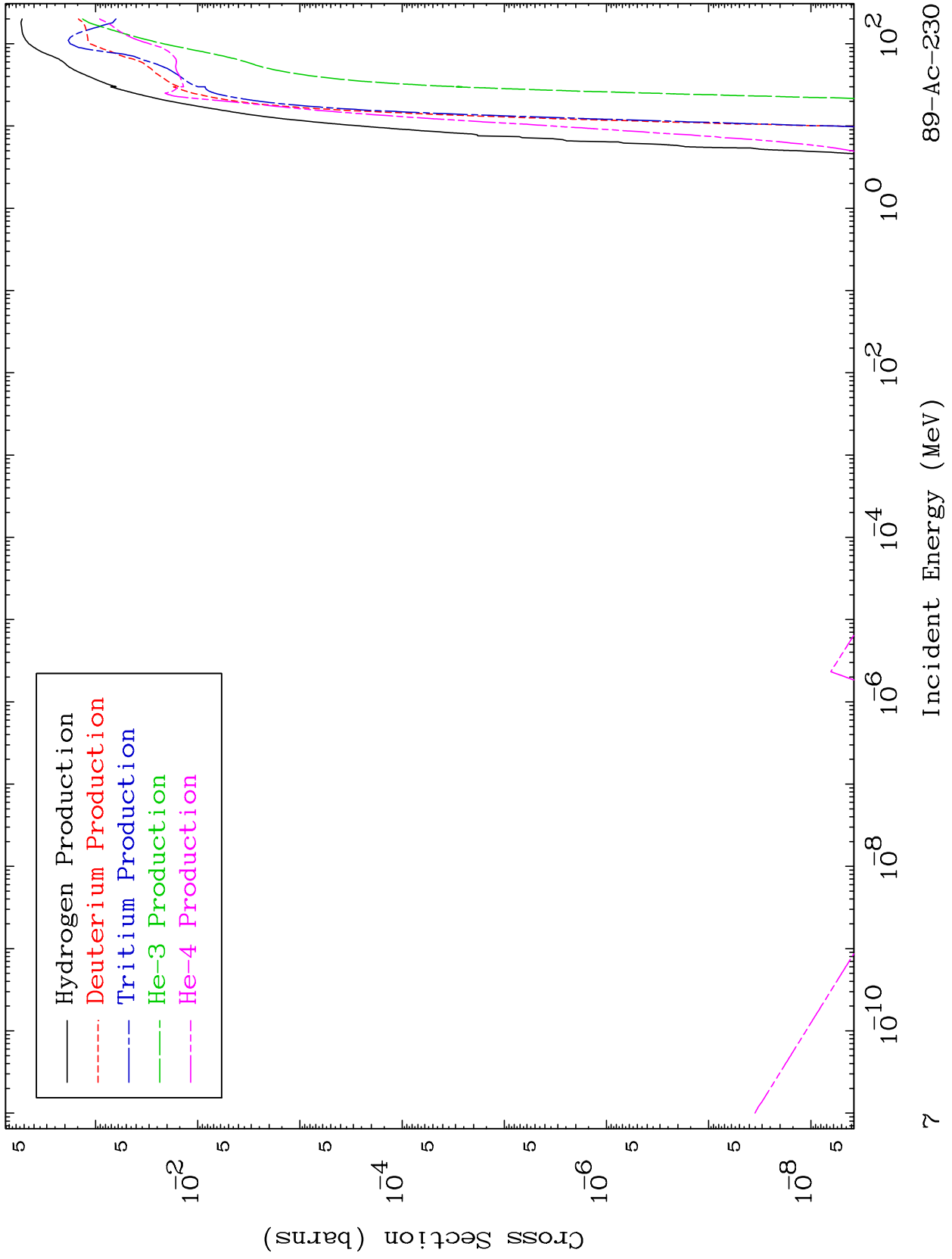
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Particle Production  
293 Kelvin Cross Sections

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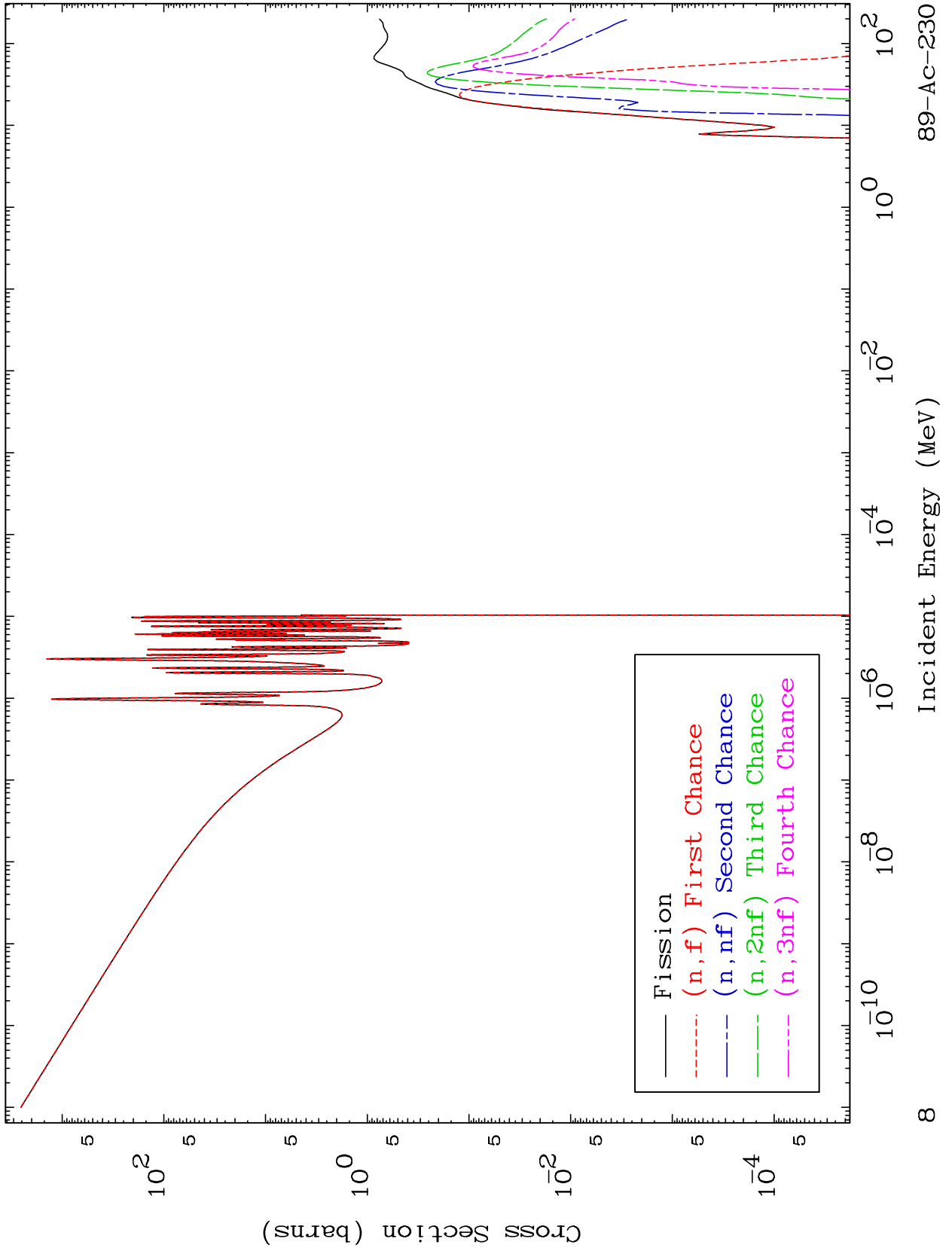




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Fission  
293 Kelvin Cross Sections

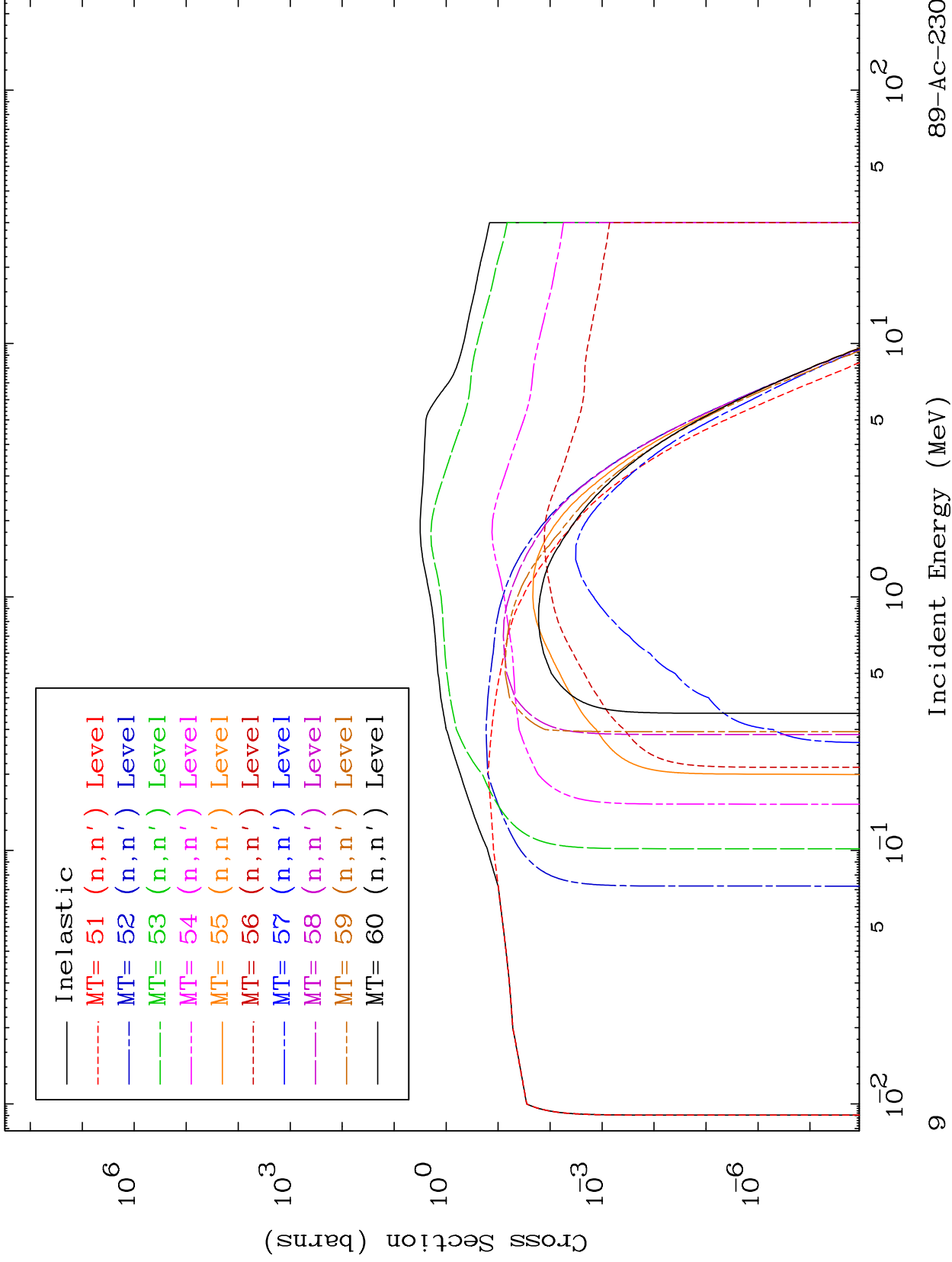
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MAT 8940

(n,n') Levels  
293 Kelvin Cross Sections

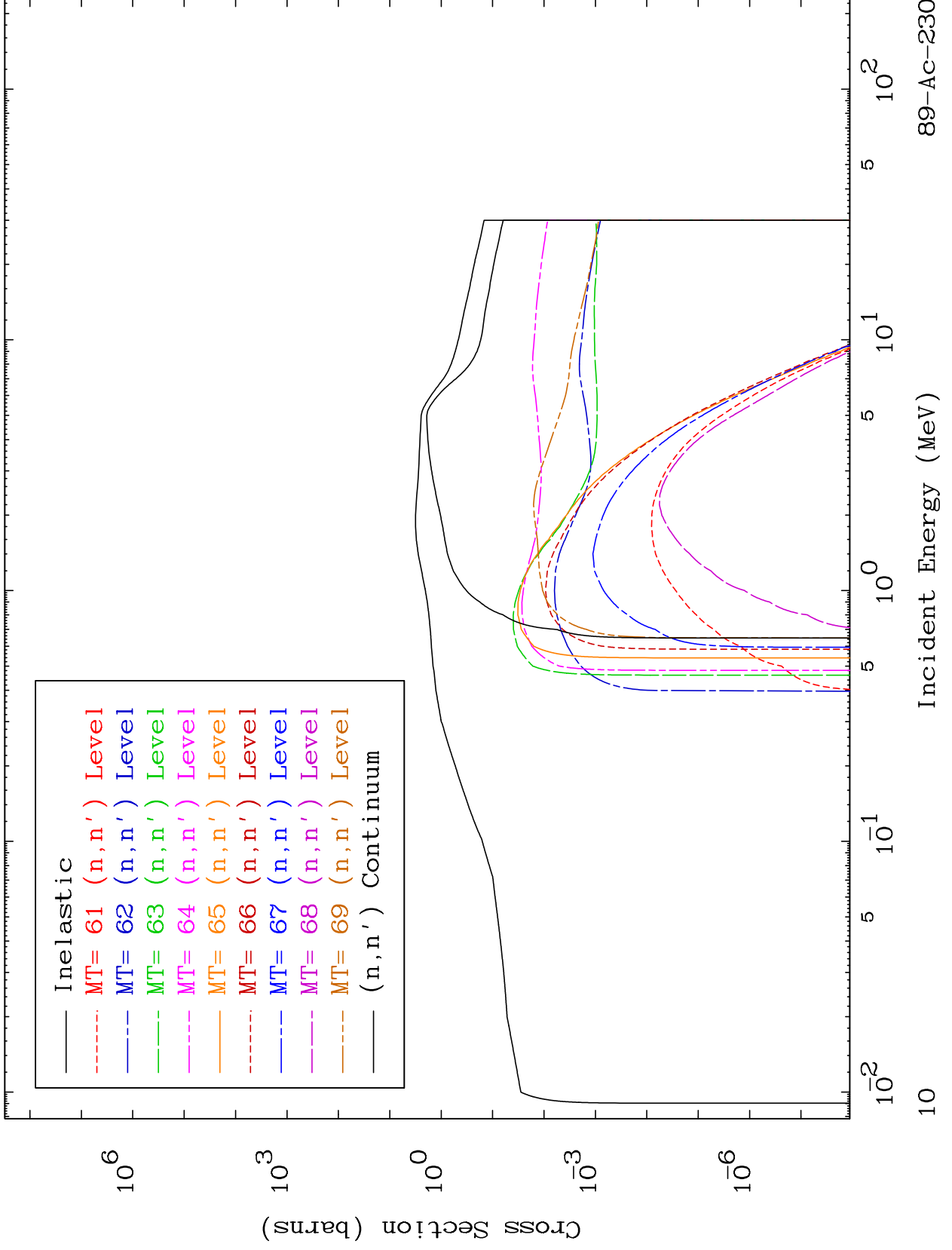
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(n,n') Levels  
293 Kelvin Cross Sections

89-Ac-230

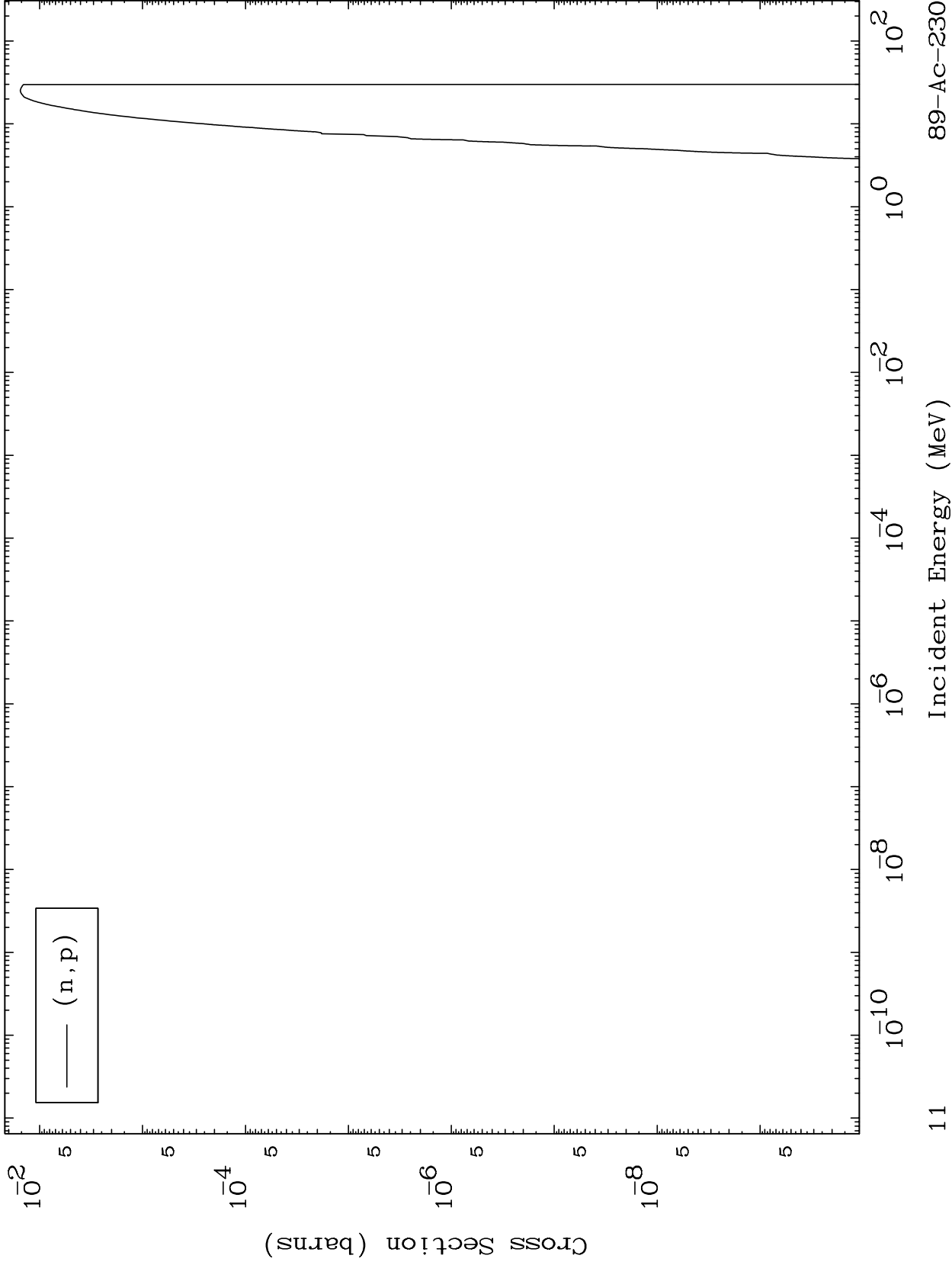


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(n,p) Levels  
293 Kelvin Cross Sections

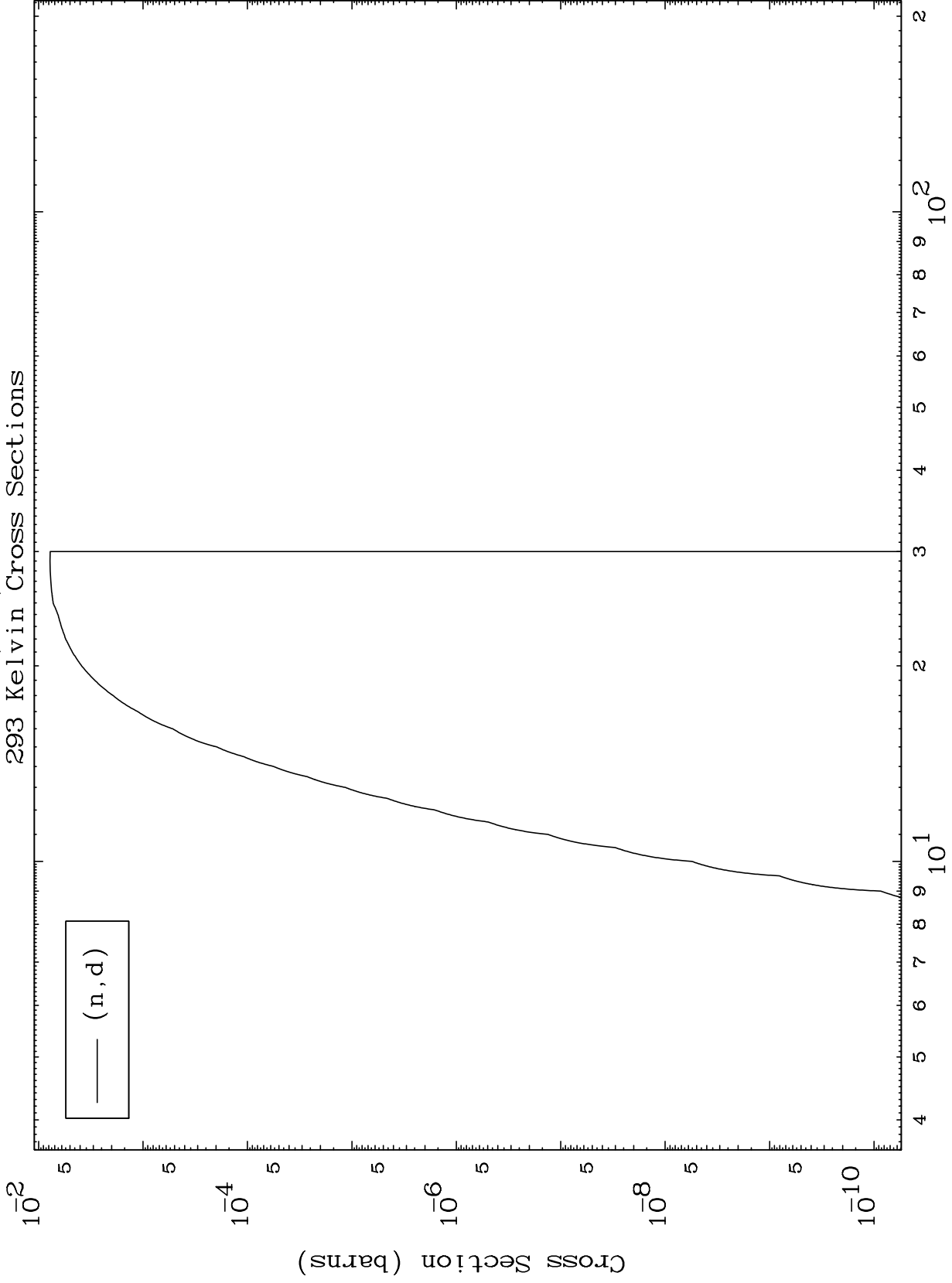
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(n,d) Levels  
293 Kelvin Cross Sections

89-Ac-230



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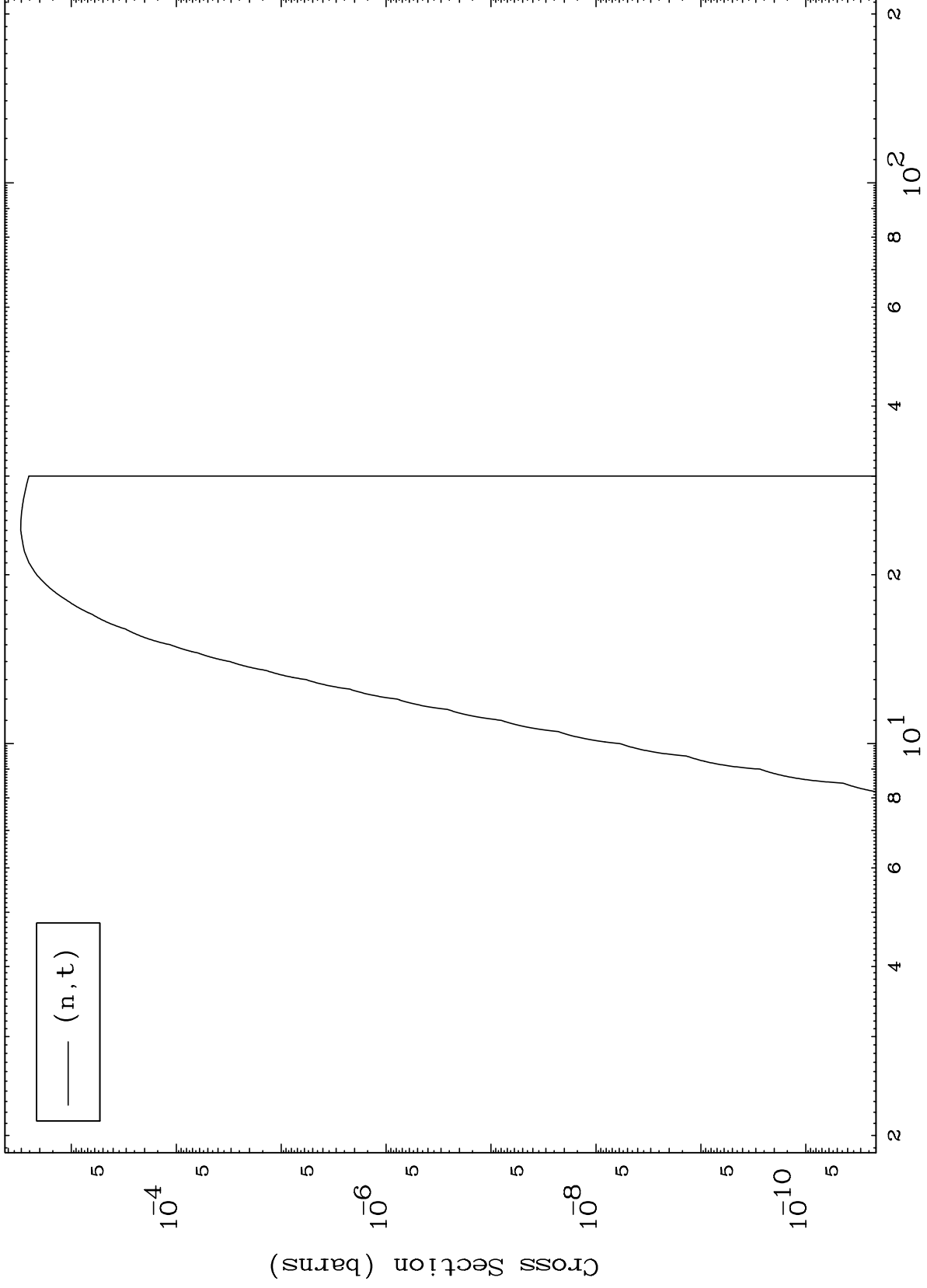
Incident Energy (MeV)

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(n,t) Levels  
293 Kelvin Cross Sections

89-Ac-230



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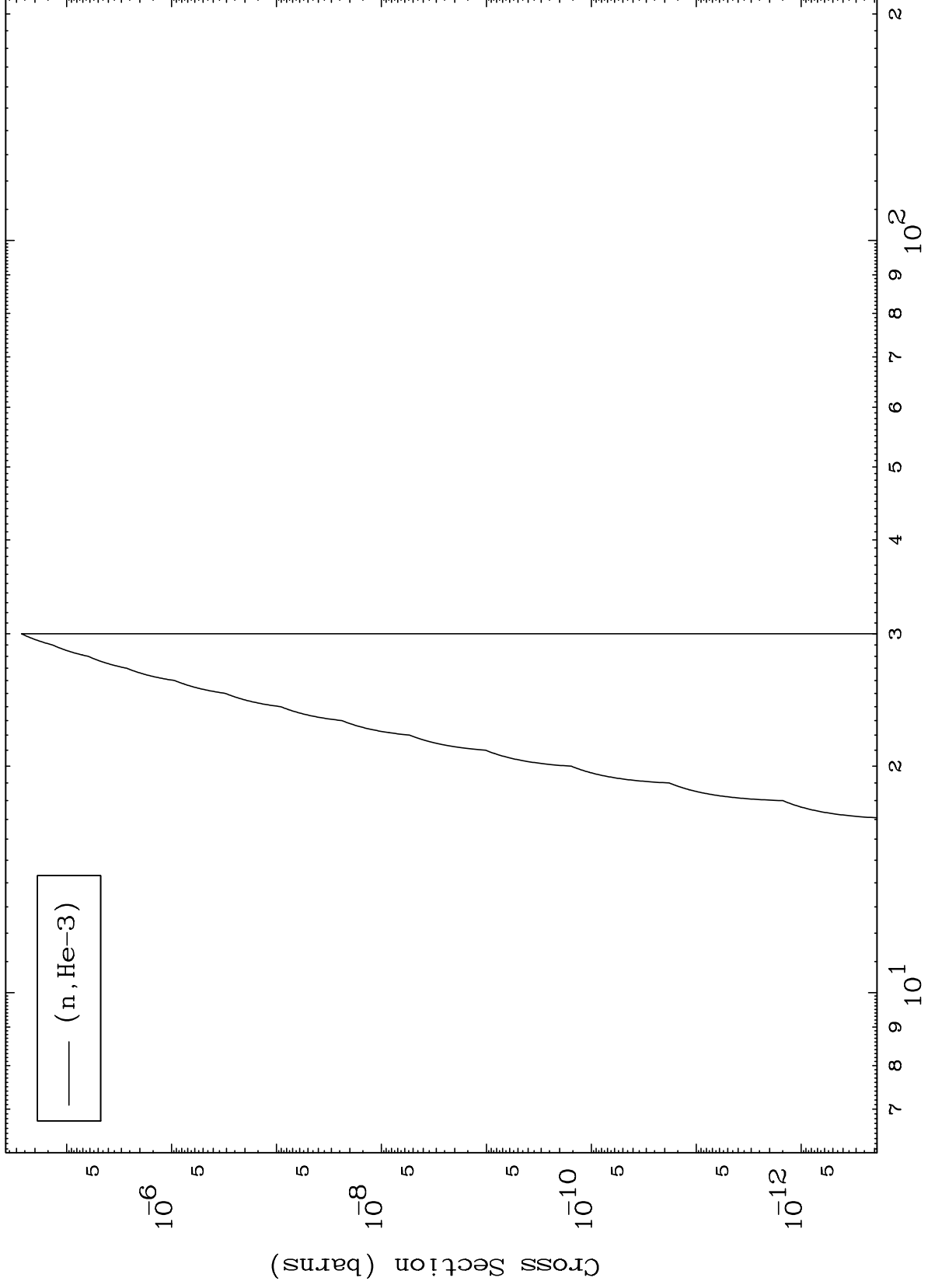
Incident Energy (MeV)

89-Ac-230

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(n,He3) Levels  
293 Kelvin Cross Sections

89-Ac-230



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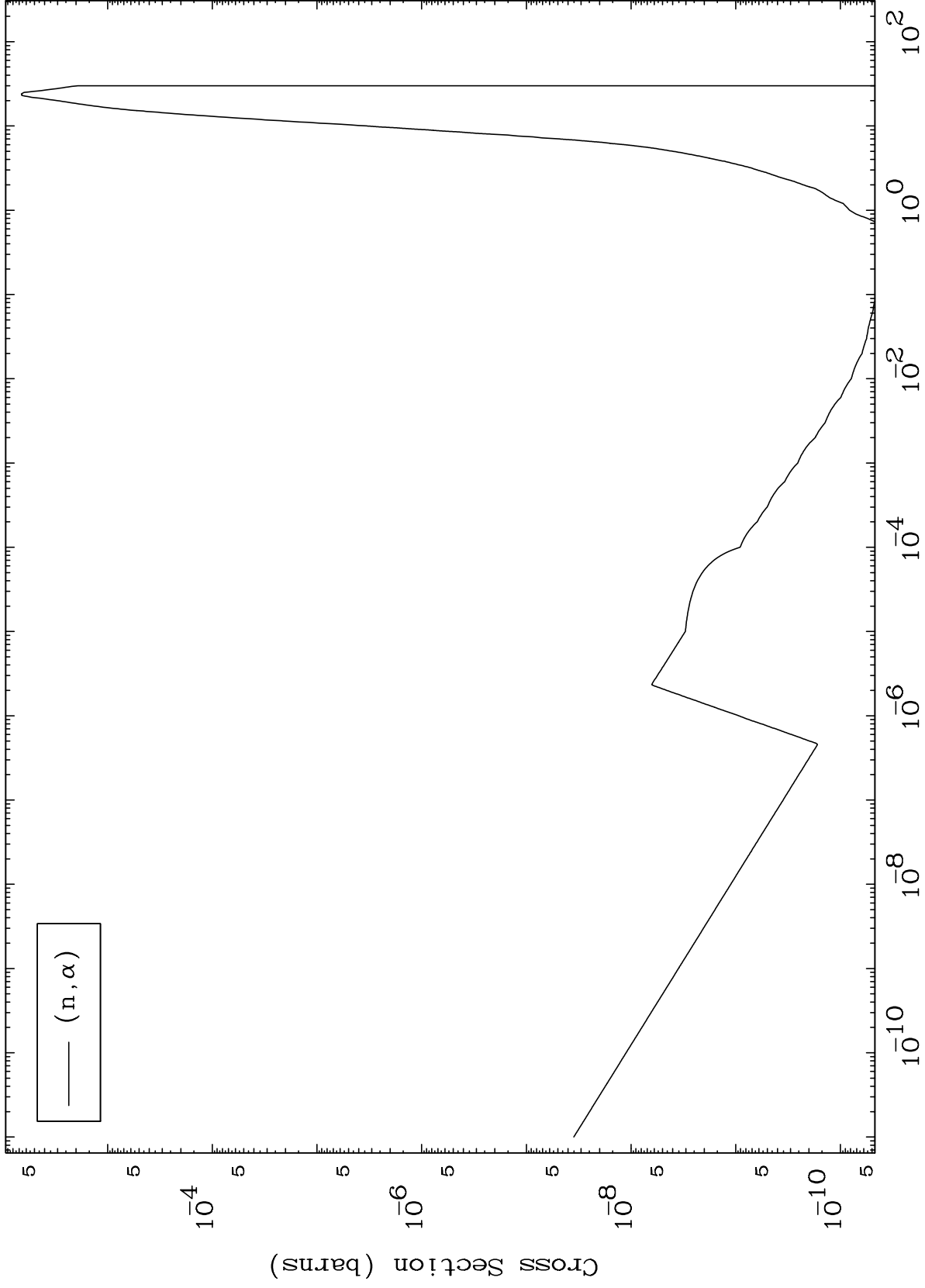
Incident Energy (MeV)

89-Ac-230

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

89-Ac-230



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Incident Energy (MeV)

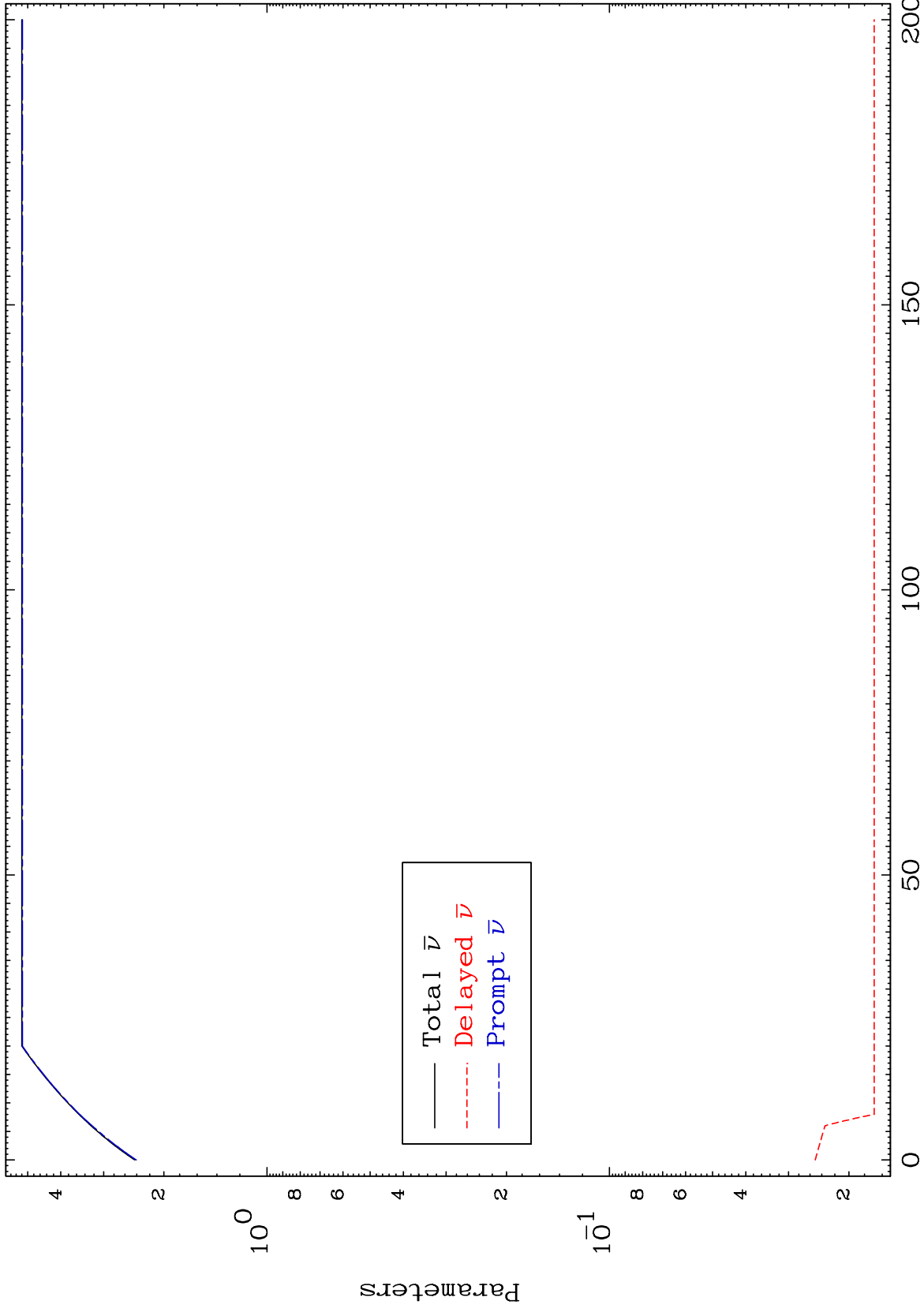
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Energy Release  
Parameters

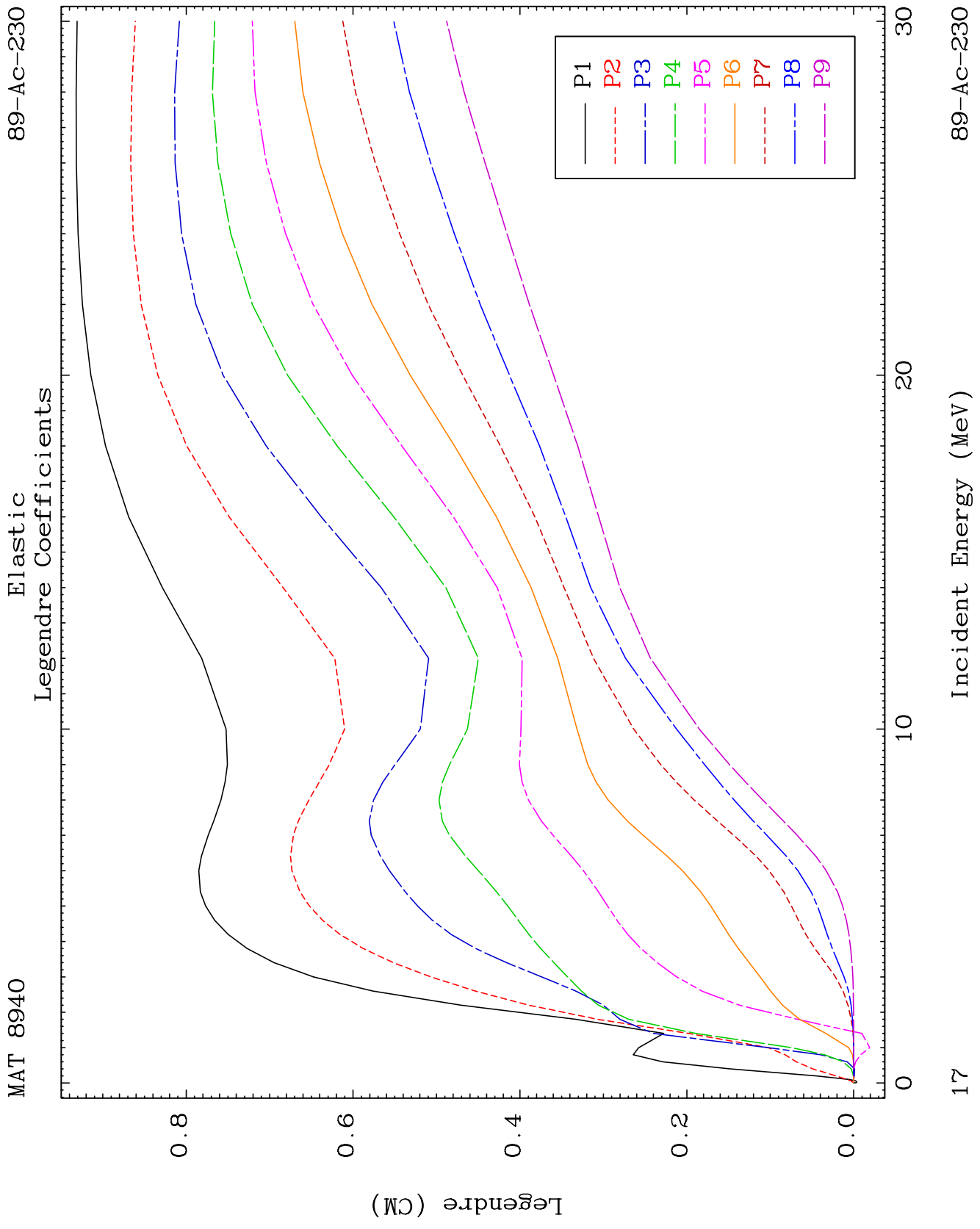
89-Ac-230



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Incident Energy (MeV)

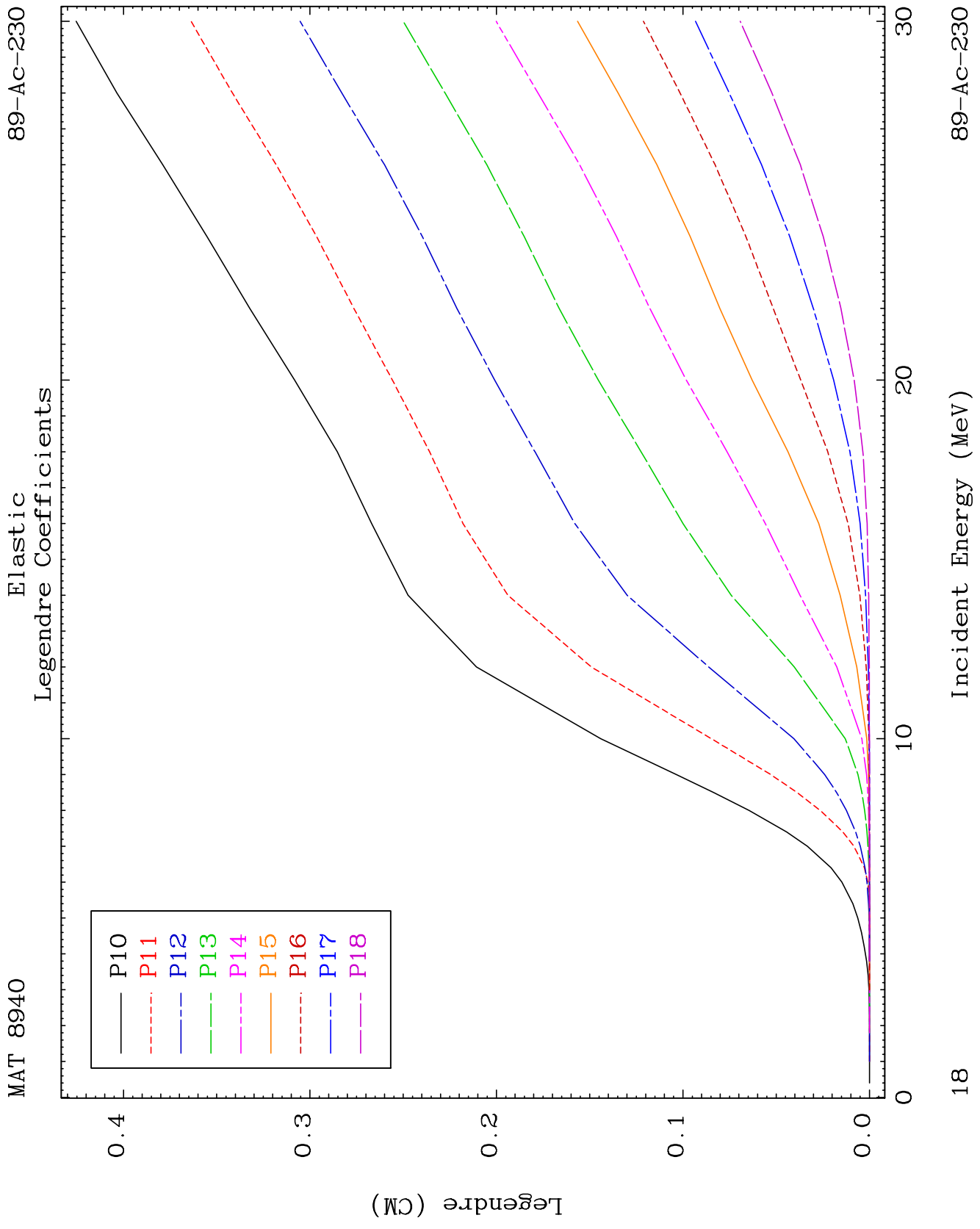
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89-Ac-230

Incident Energy (MeV)

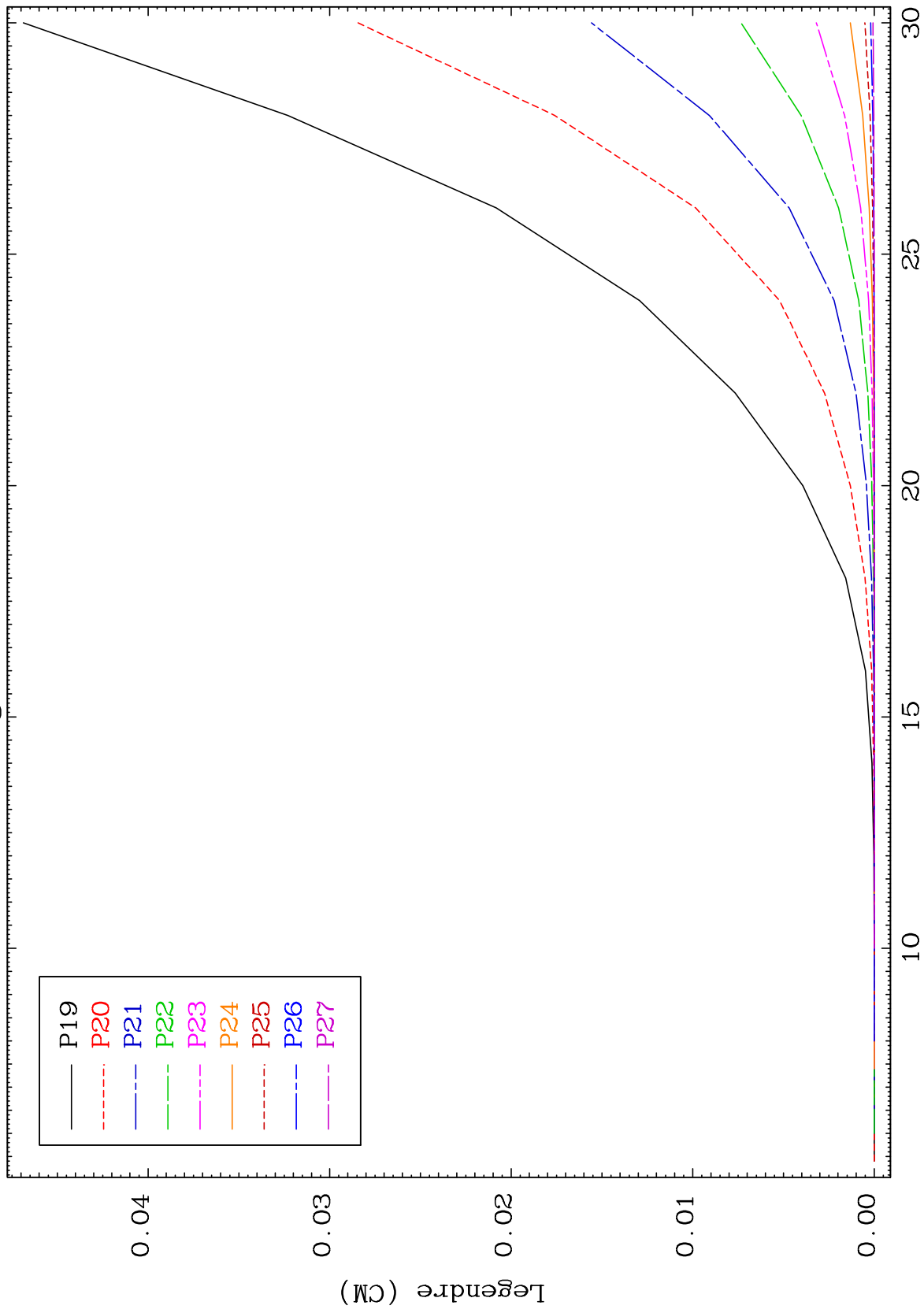
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### Elastic Legendre Coefficients

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Incident Energy (MeV)

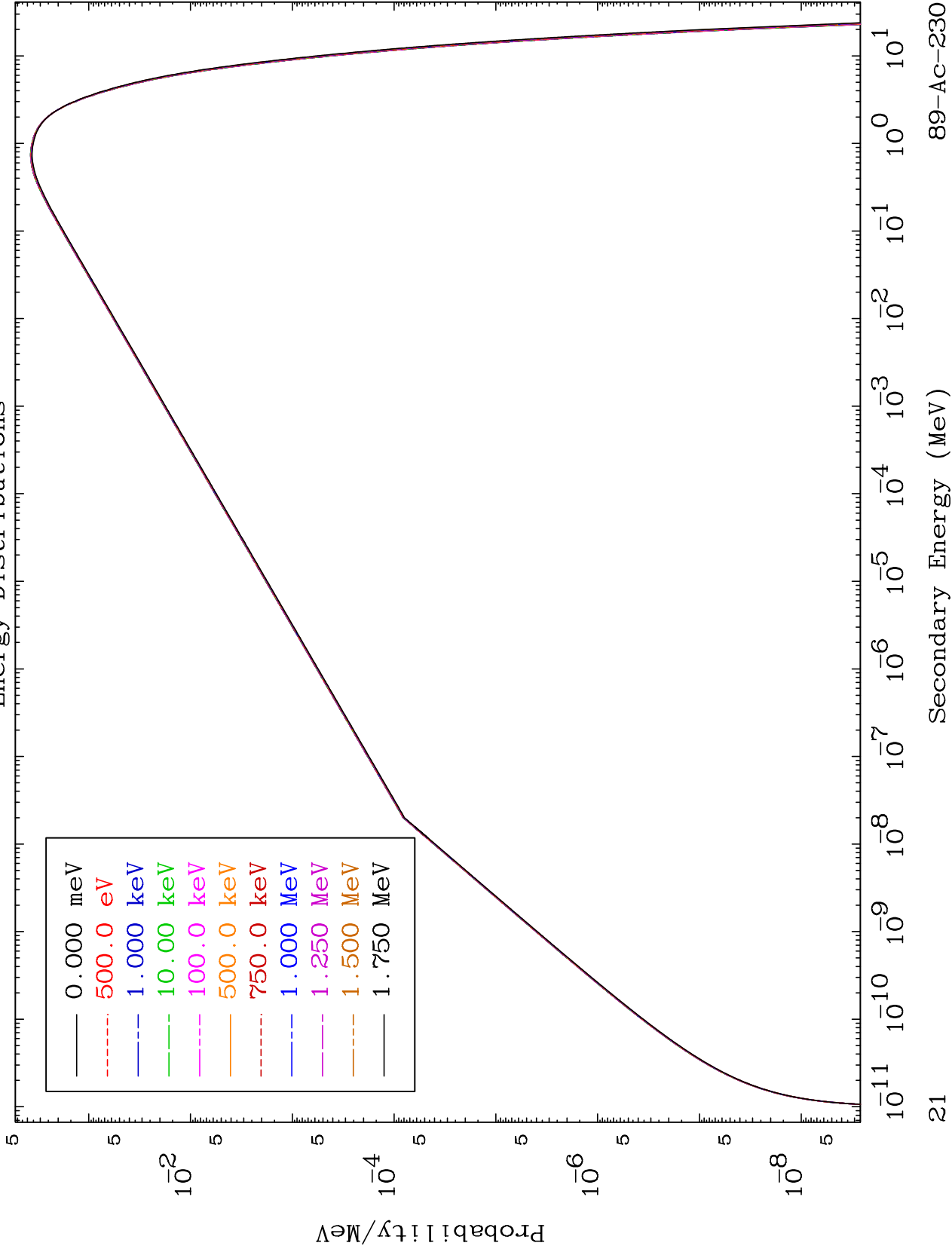
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### Fission Energy Distributions

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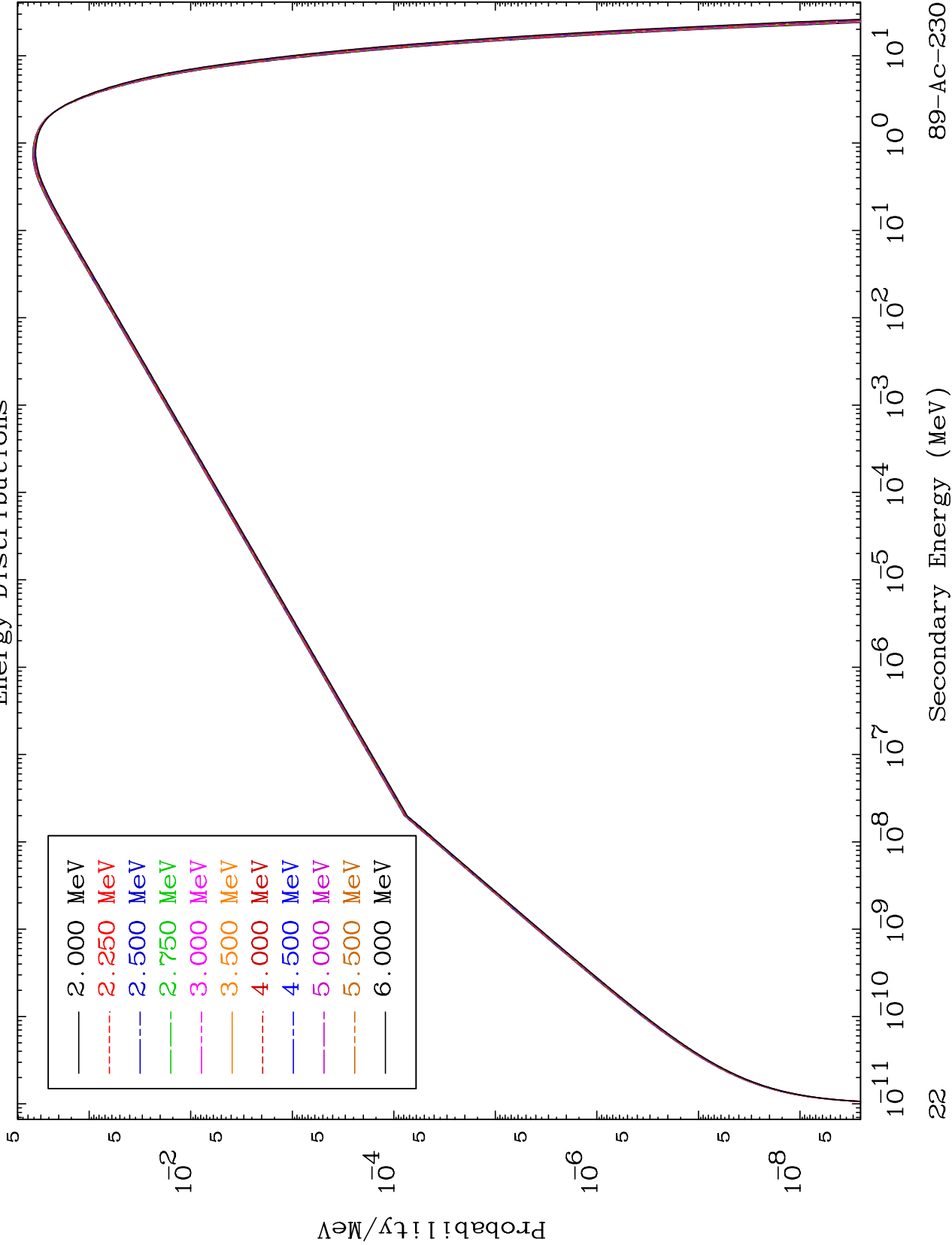
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Secondary Energy (MeV)

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### Fission Energy Distributions

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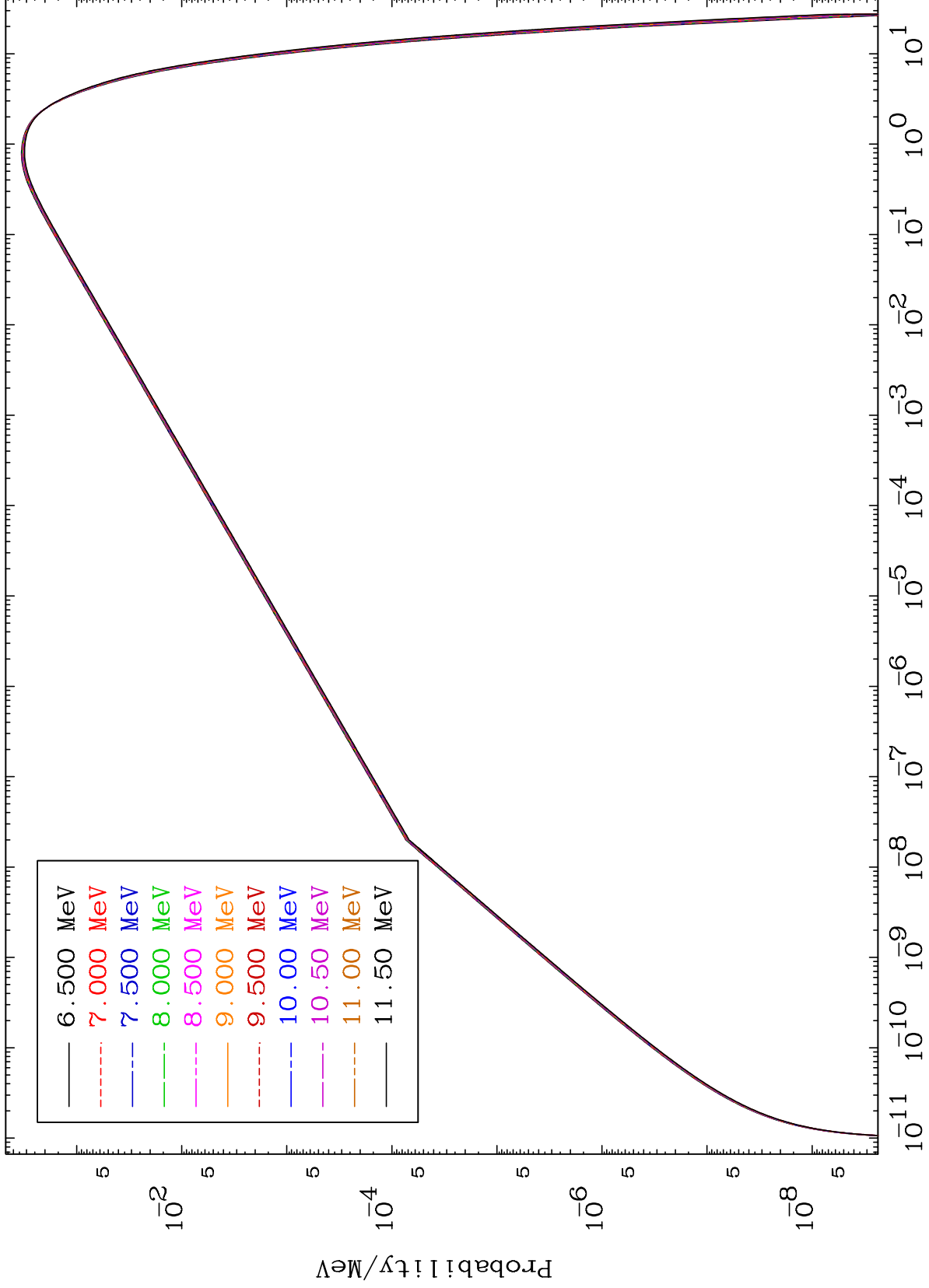
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Secondary Energy (MeV)

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### Fission Energy Distributions

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Secondary Energy (MeV)

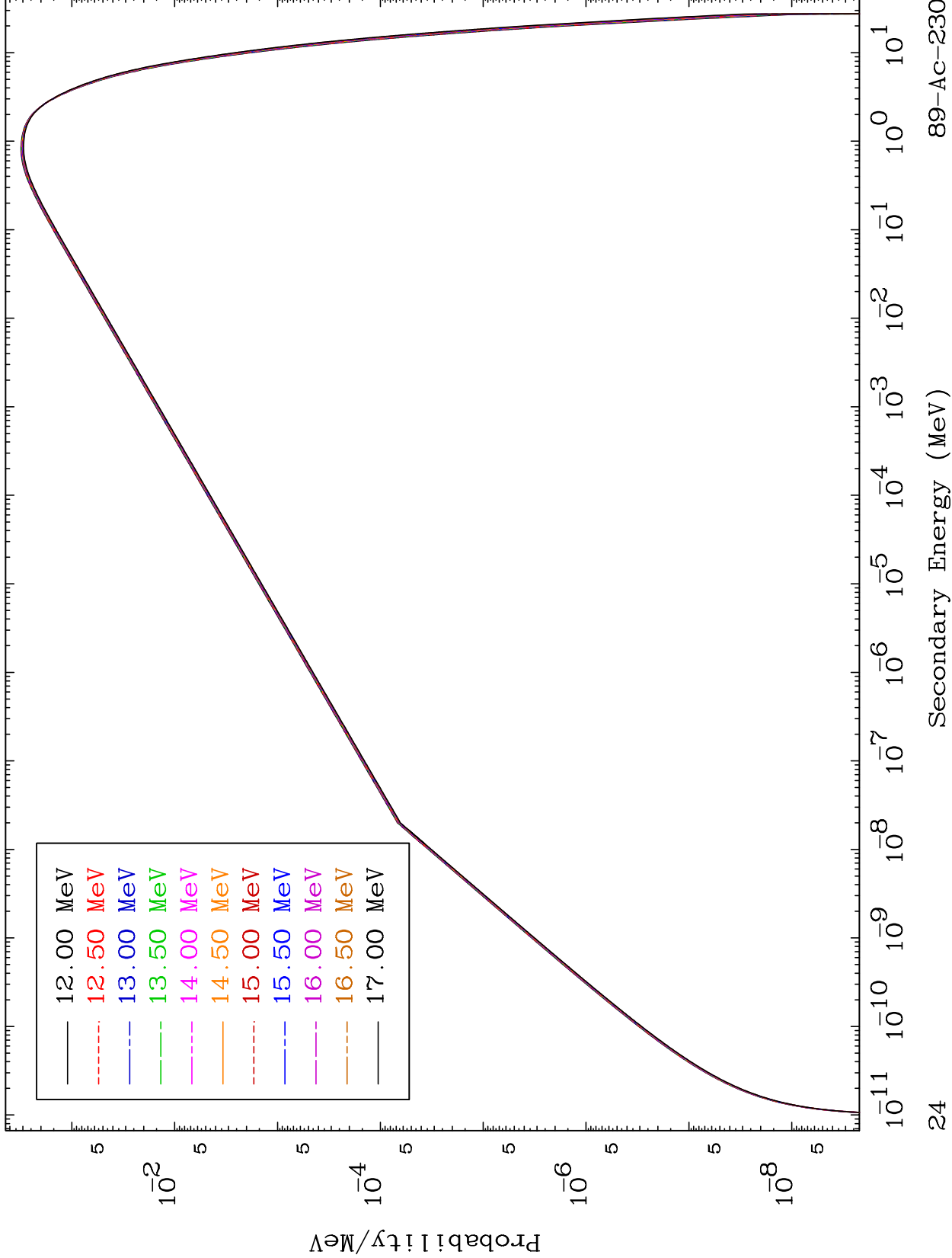
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### Fission Energy Distributions

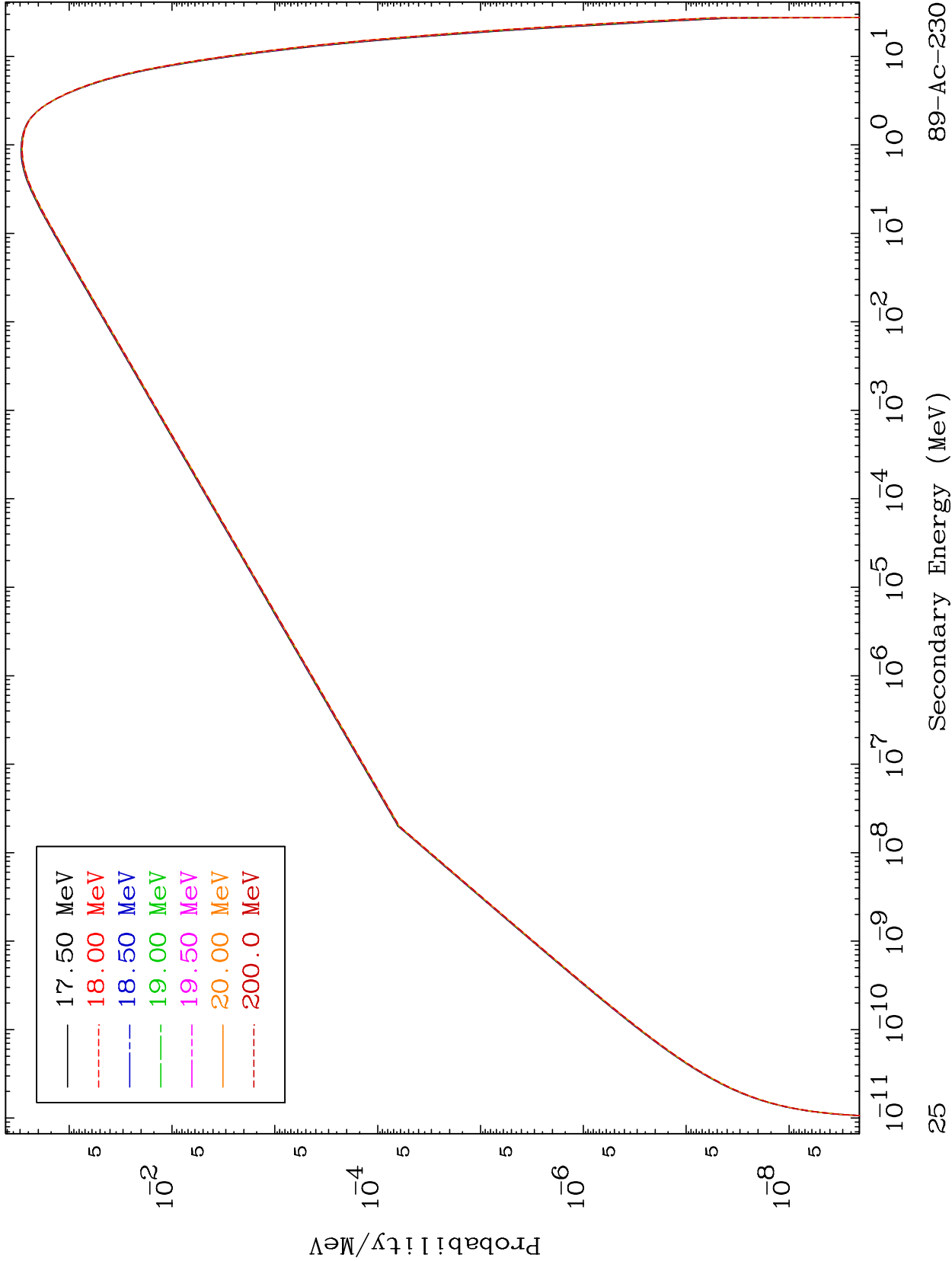
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### Fission Energy Distributions

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(n,3n)  $\alpha$

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Radionuclide Production Cross Section

