

Program EVALPLOT  
(Version 2021-1)

by

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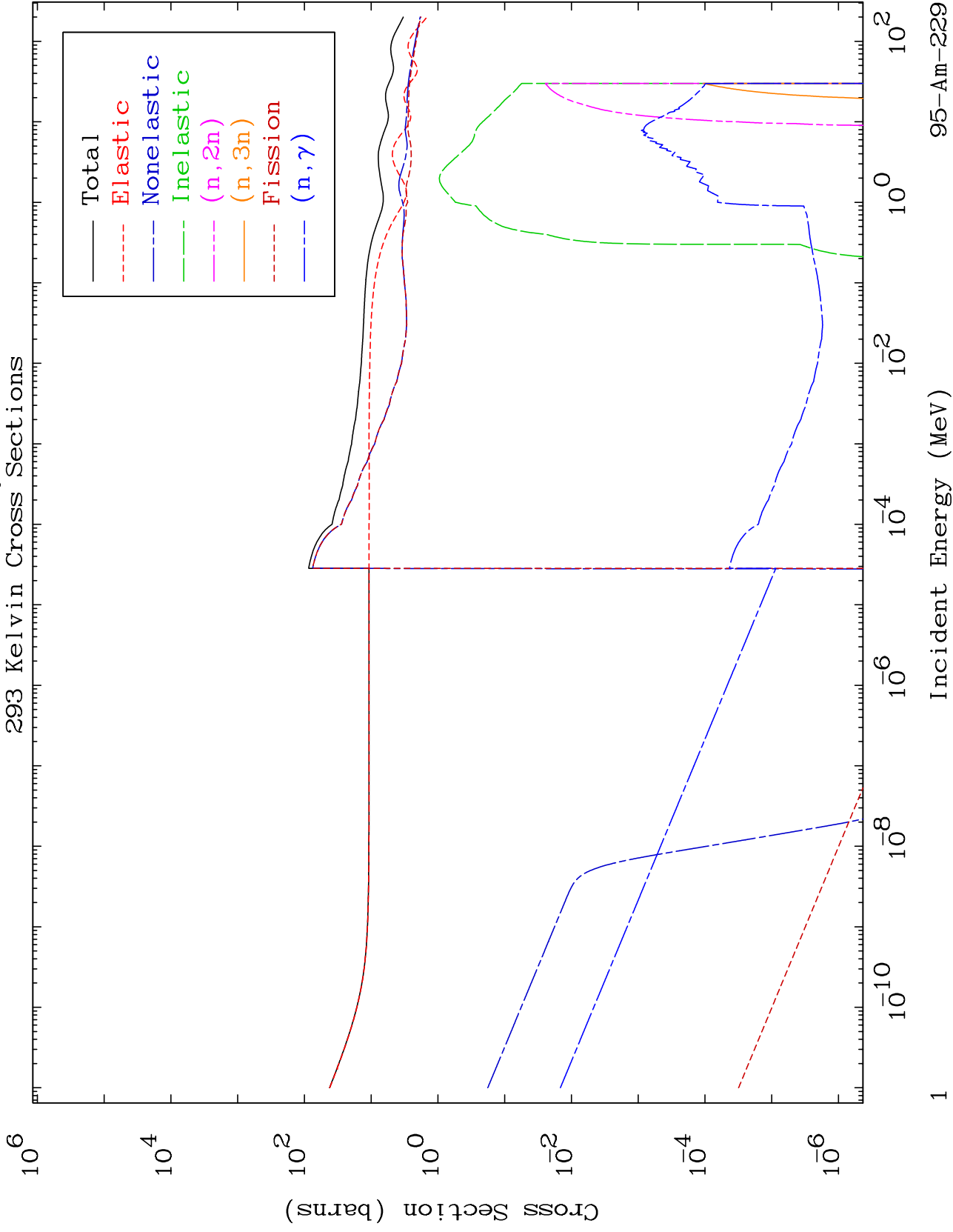
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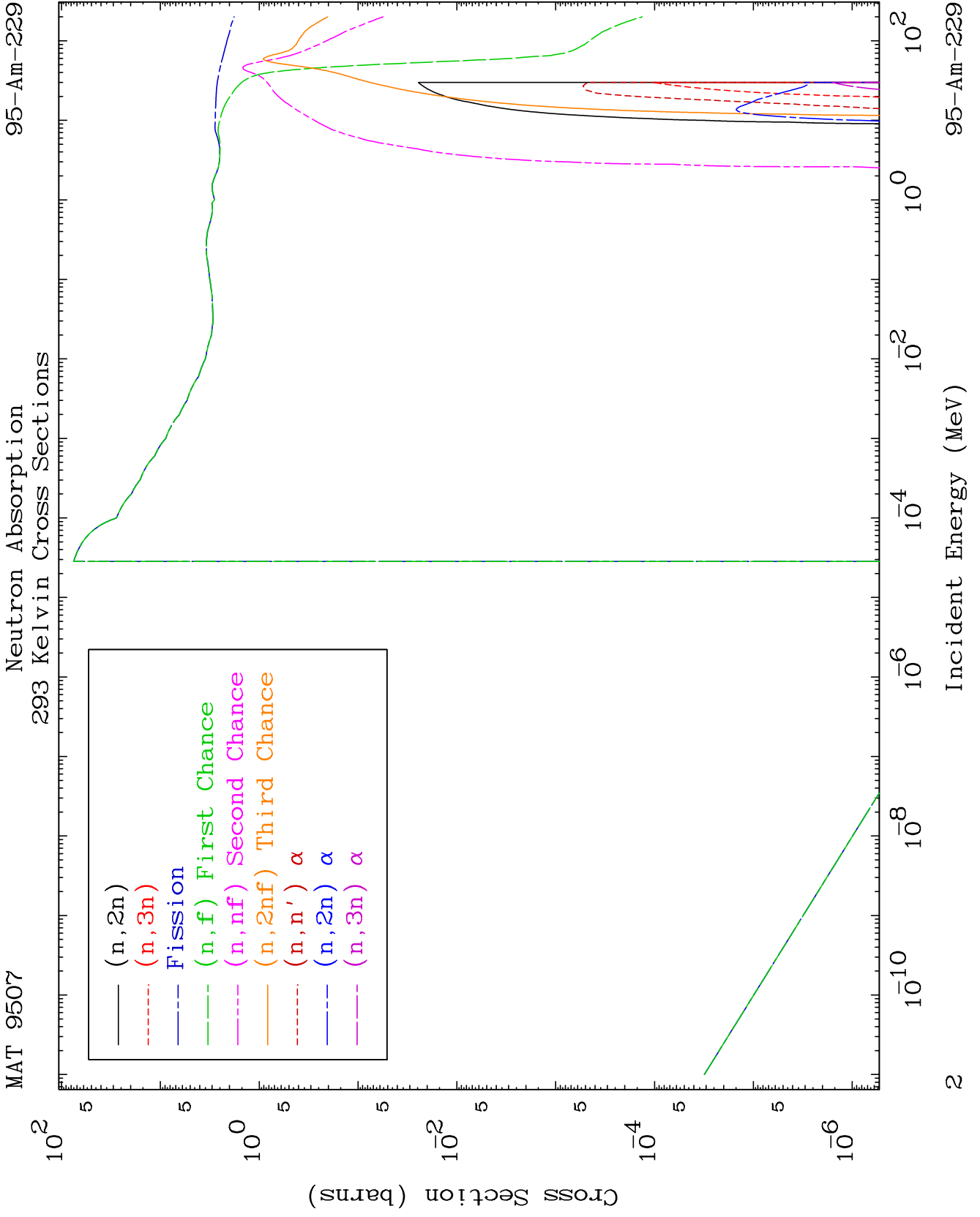
Press Mouse Button to Start

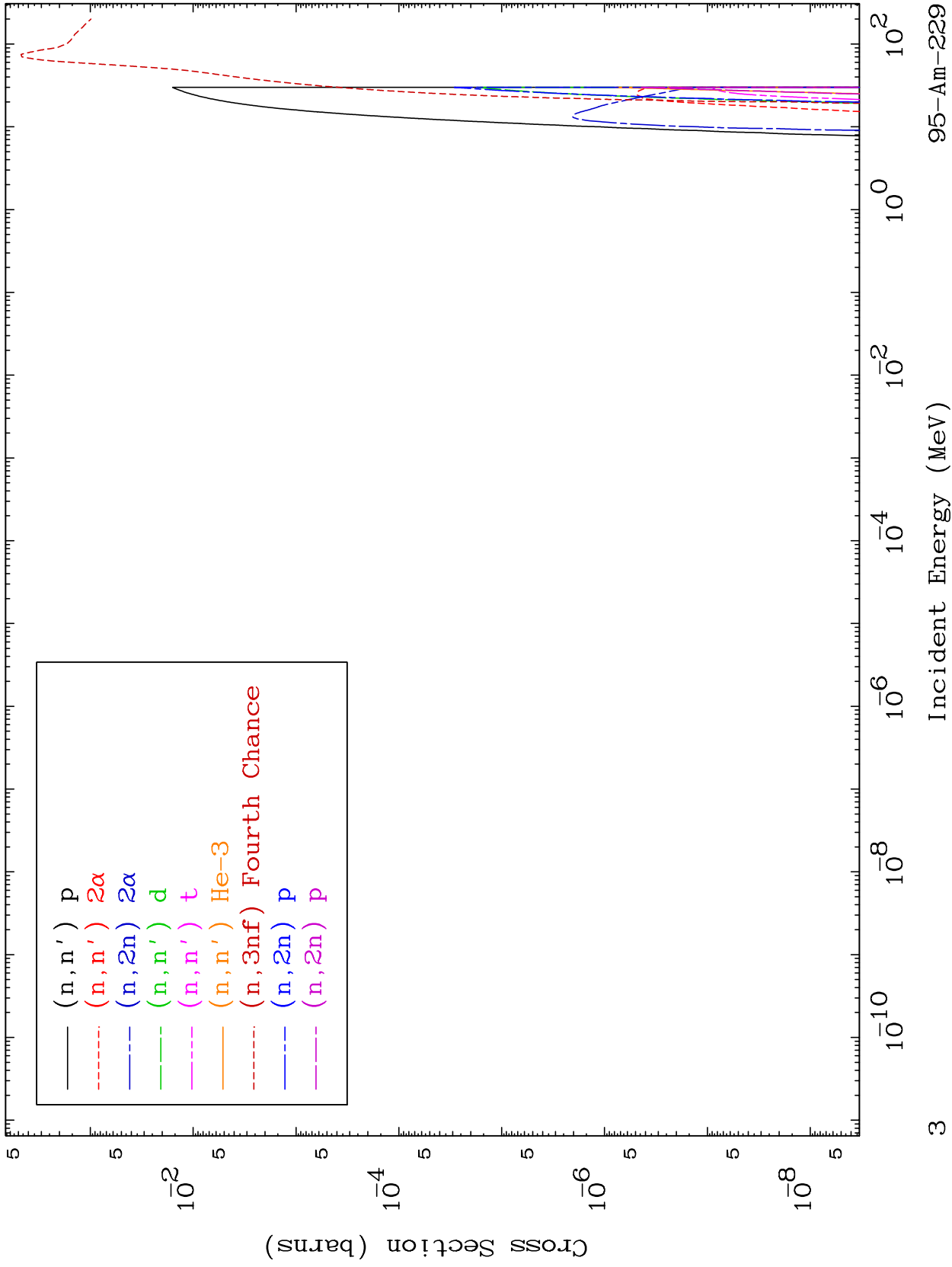
MAT 9507

Neutron Major  
293 Kelvin Cross Sections

95-Am-229



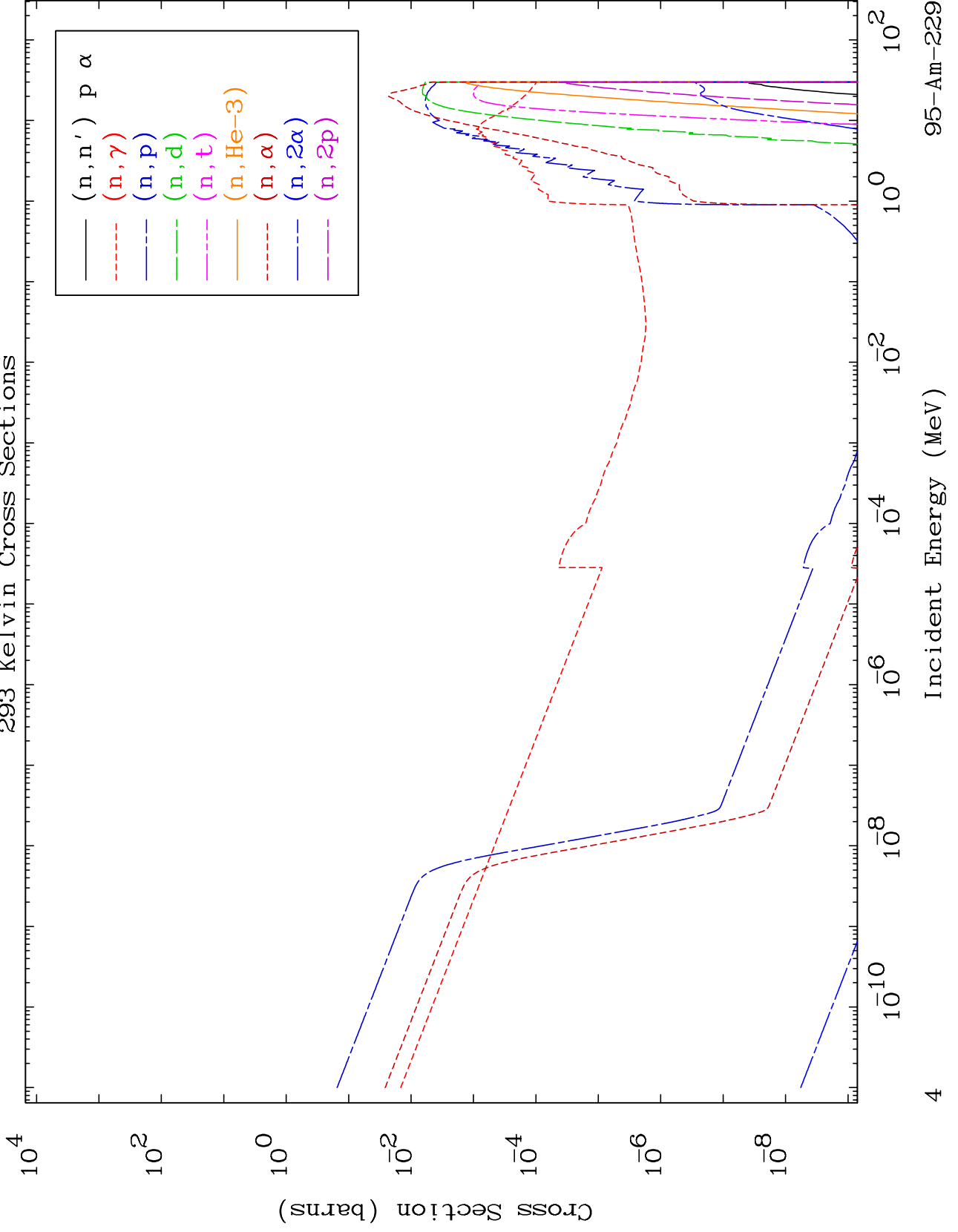




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Neutron Absorption  
293 Kelvin Cross Sections

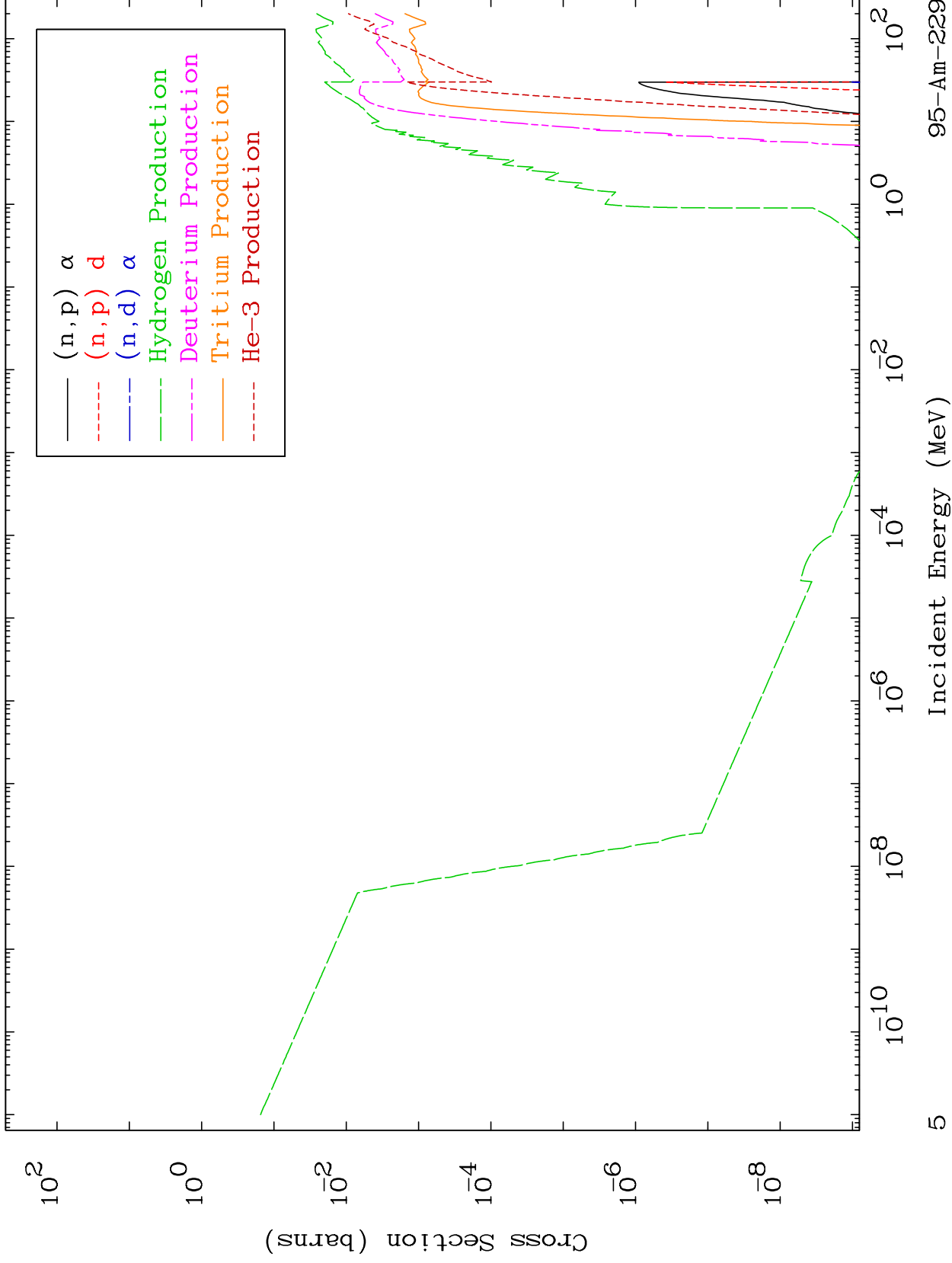
95-Am-229



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Neutron Absorption  
293 Kelvin Cross Sections

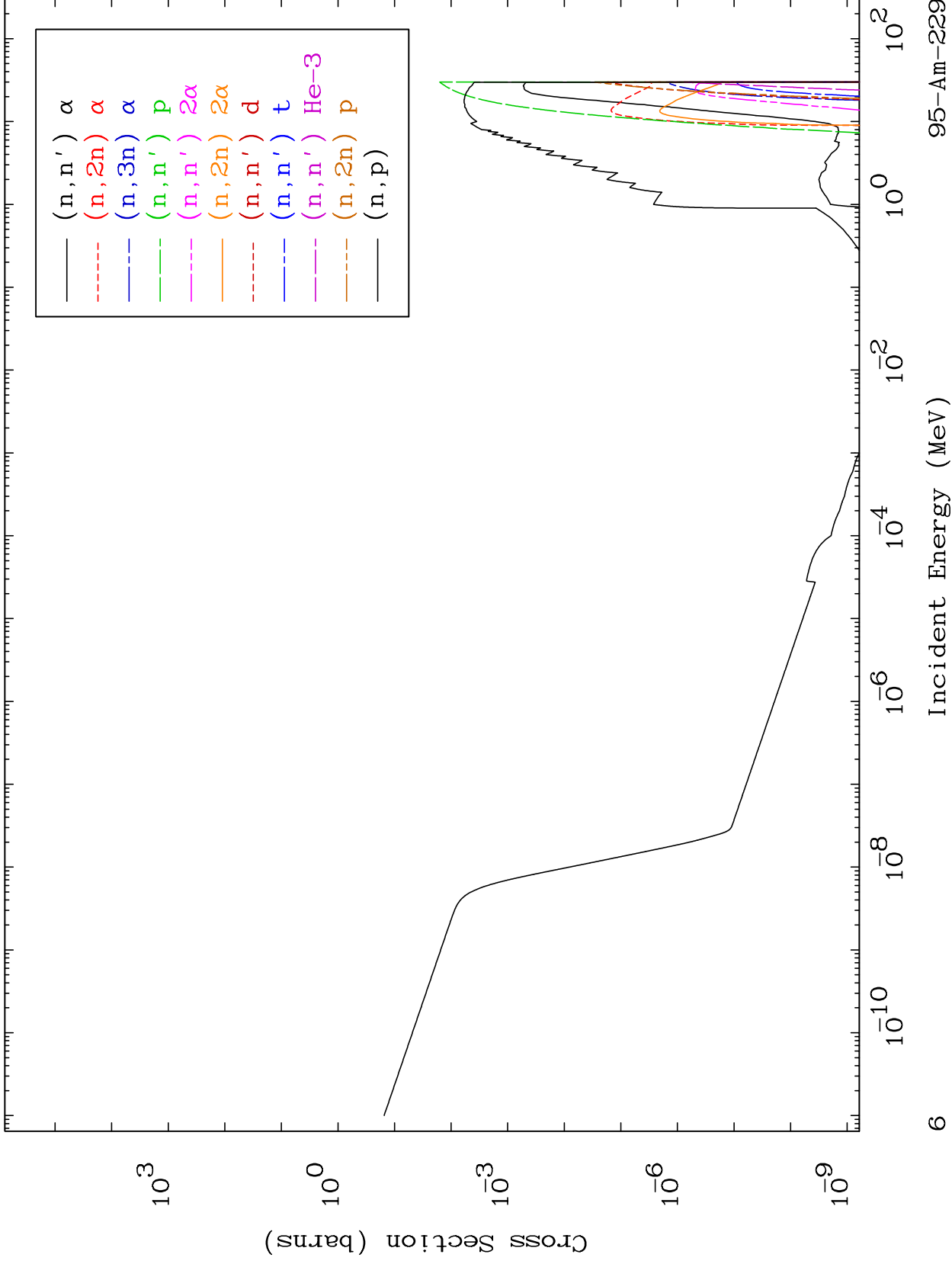
95-Am-229



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Charged Particle  
293 Kelvin Cross Sections

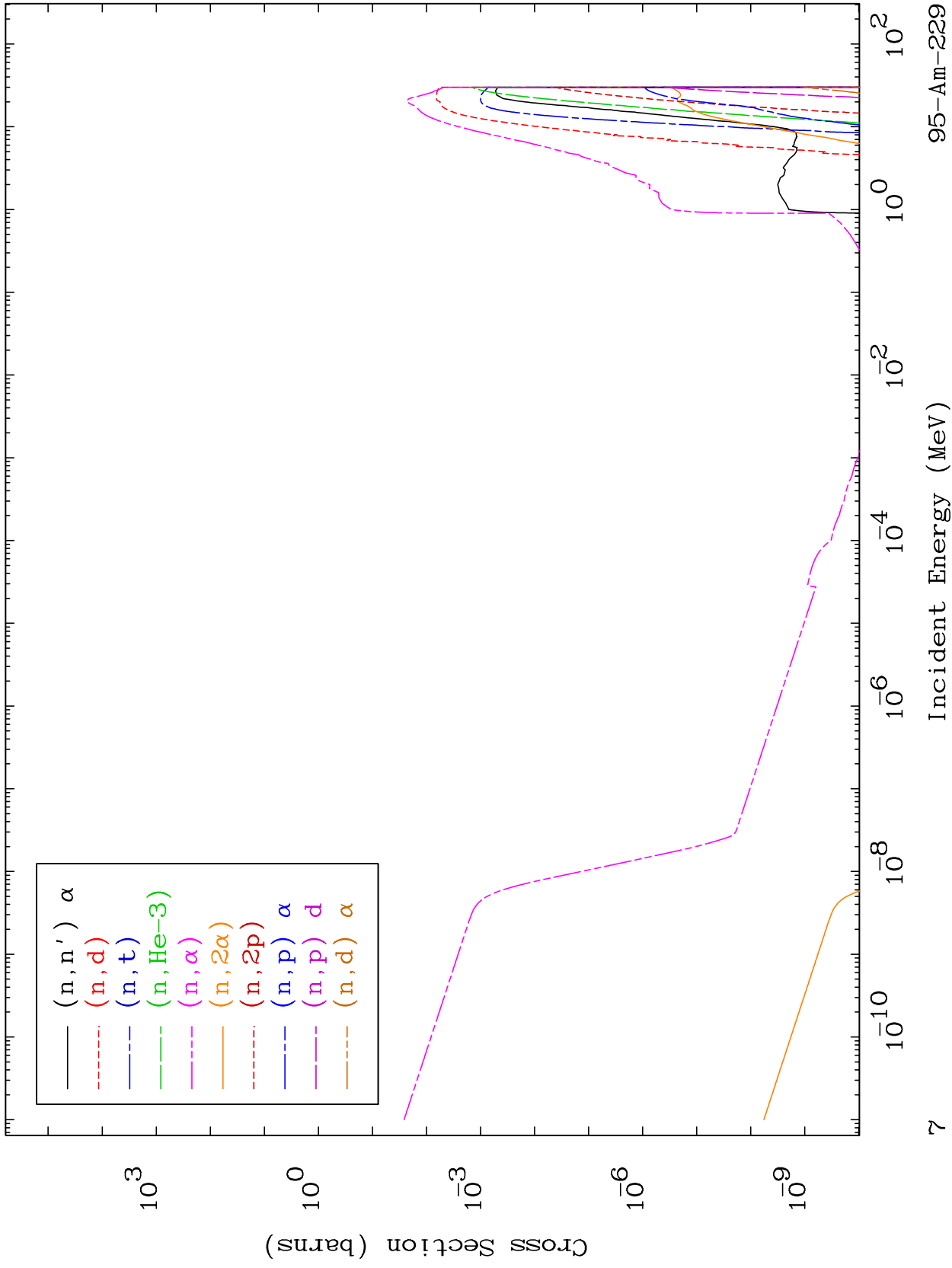
95-Am-229



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Charged Particle  
293 Kelvin Cross Sections

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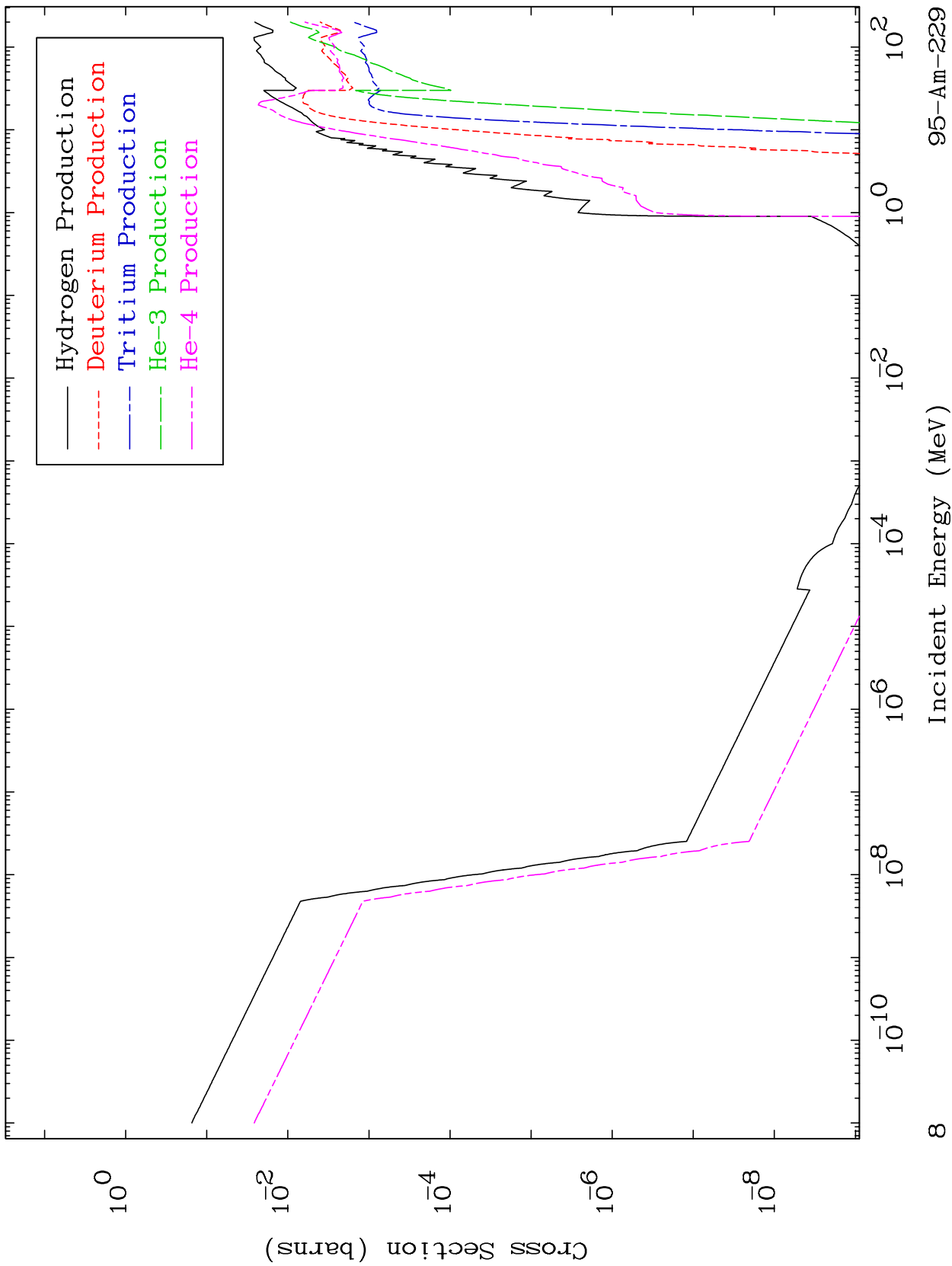


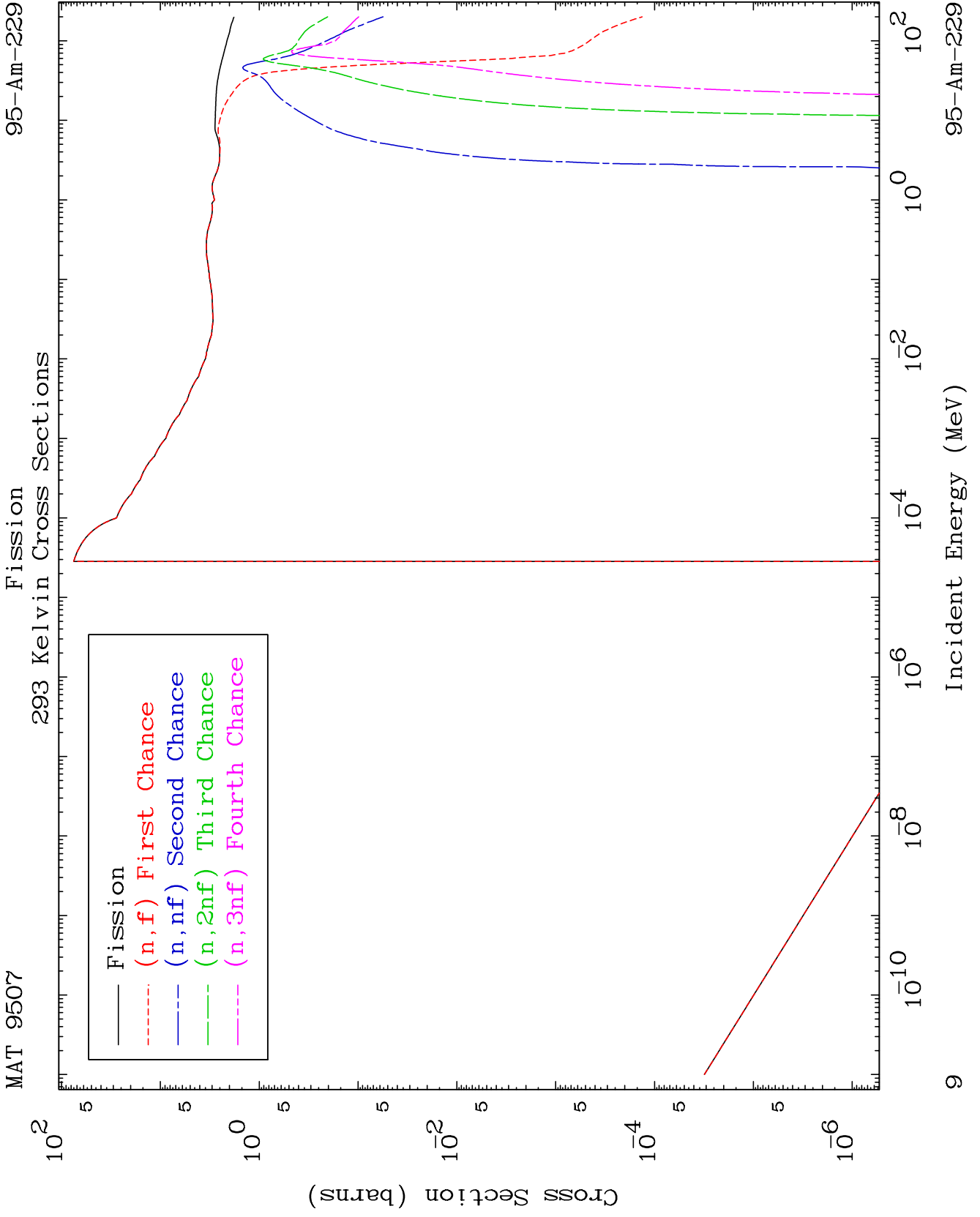


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Particle Production  
293 Kelvin Cross Sections

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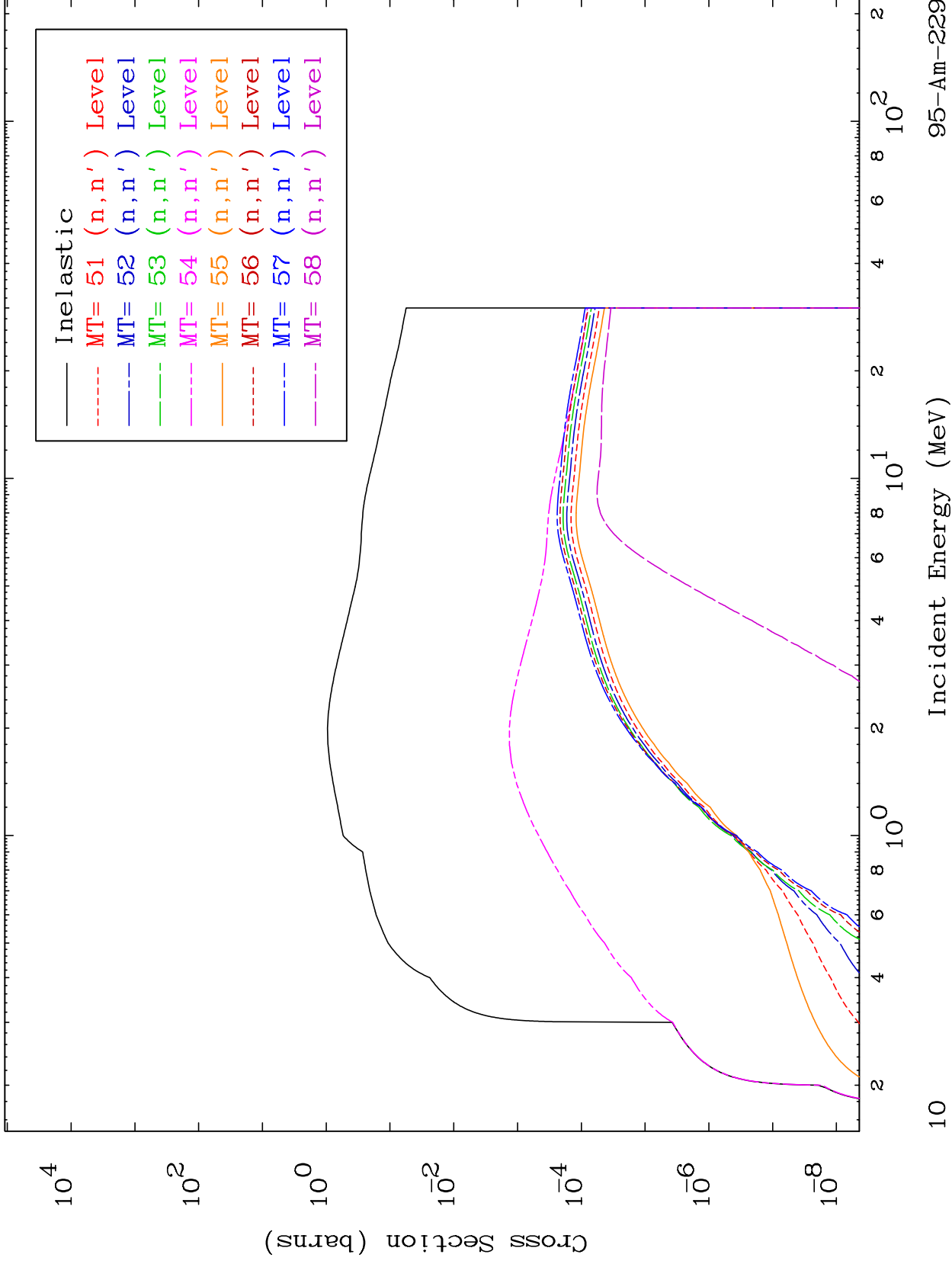


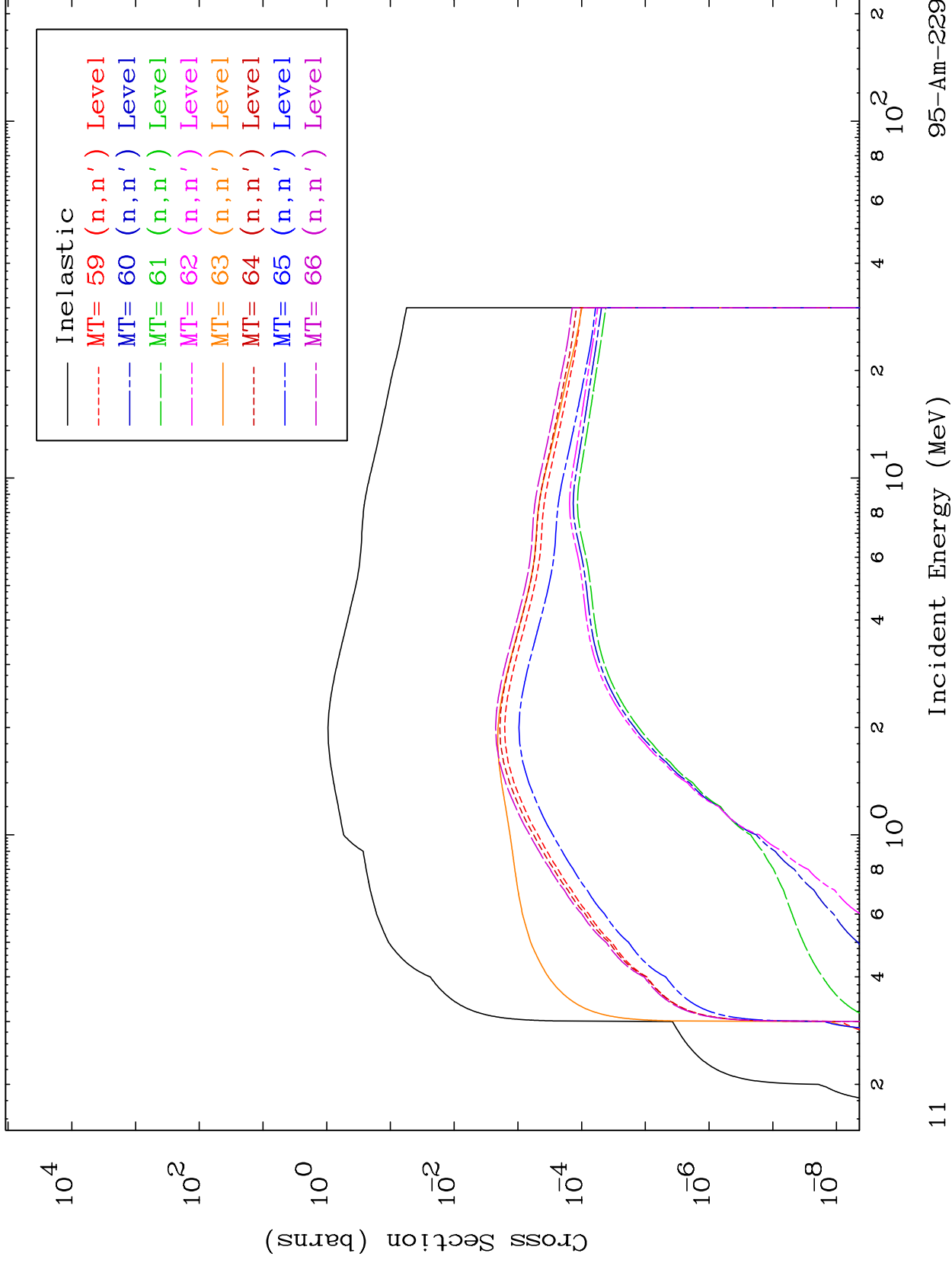


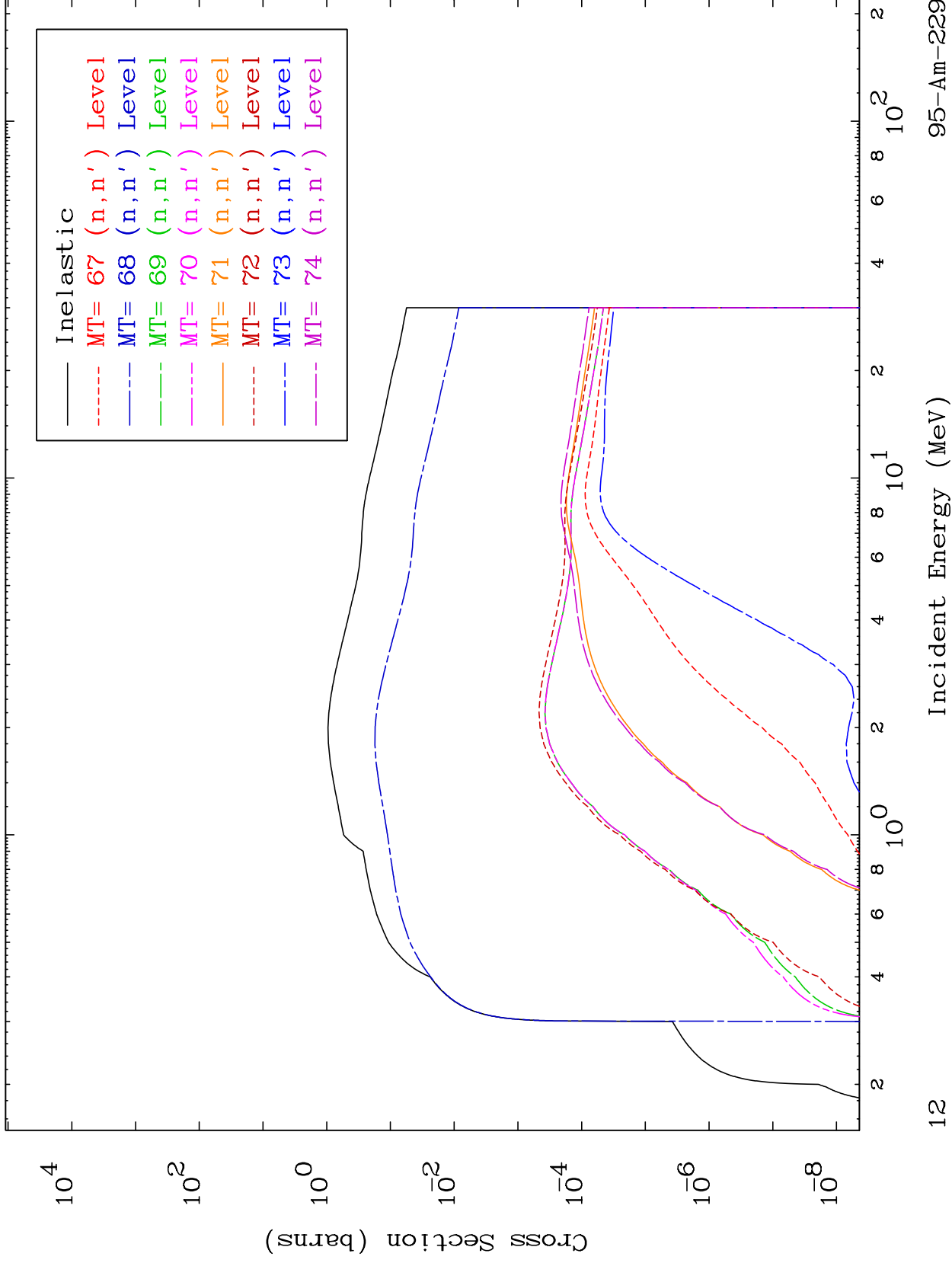
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(n,n') Levels  
293 Kelvin Cross Sections

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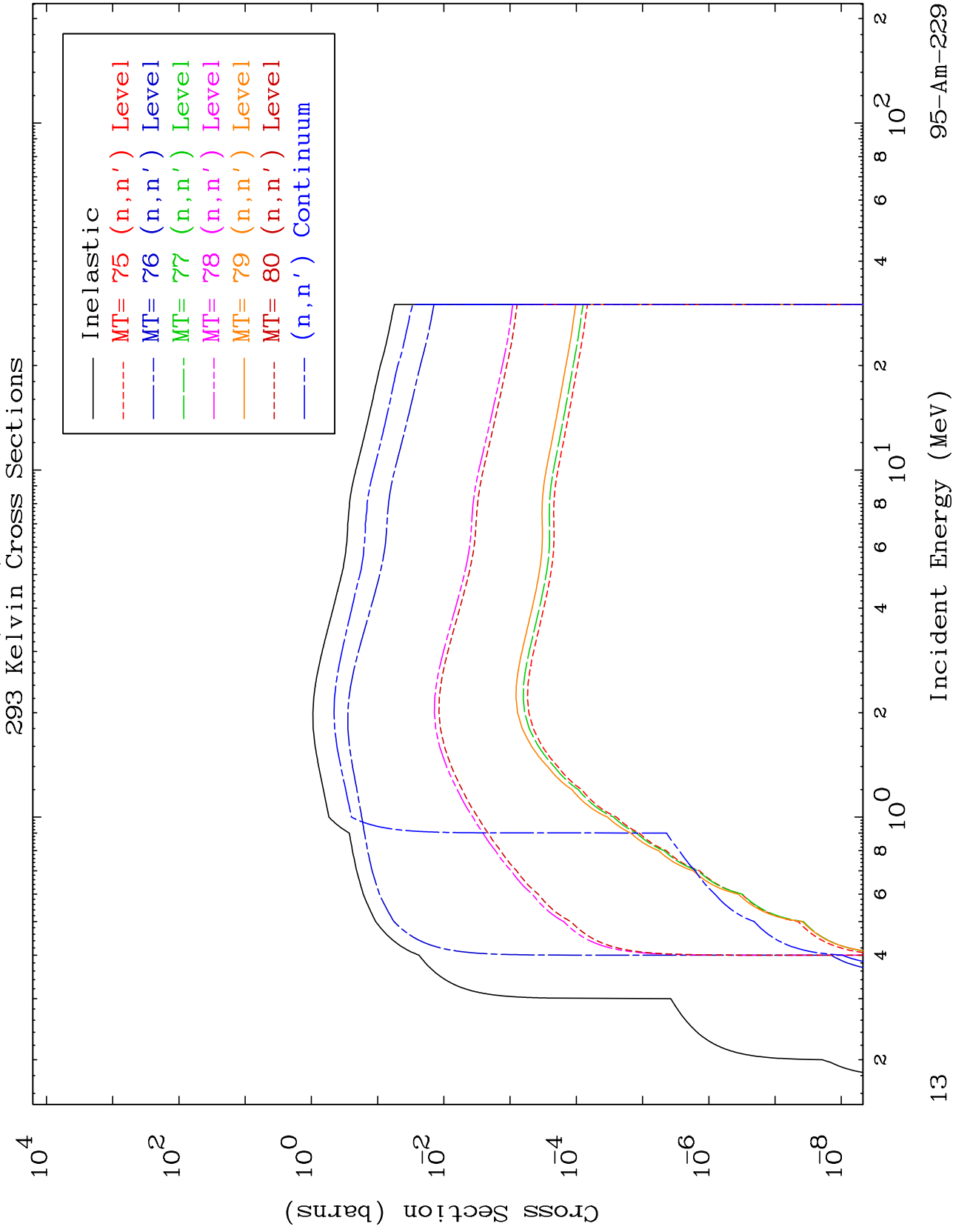




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(n,n') Levels

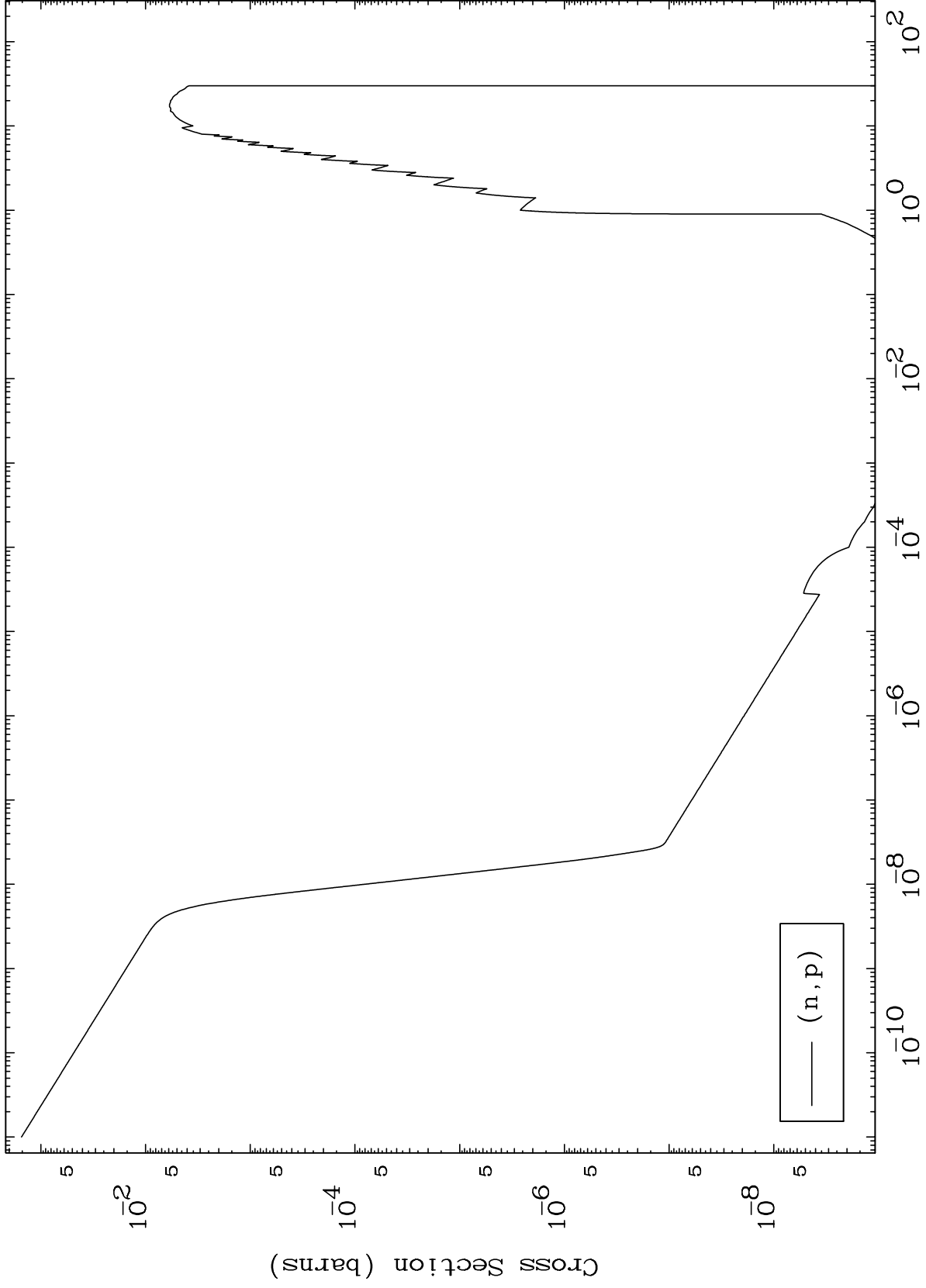
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(n,p) Levels  
293 Kelvin Cross Sections

95-Am-229



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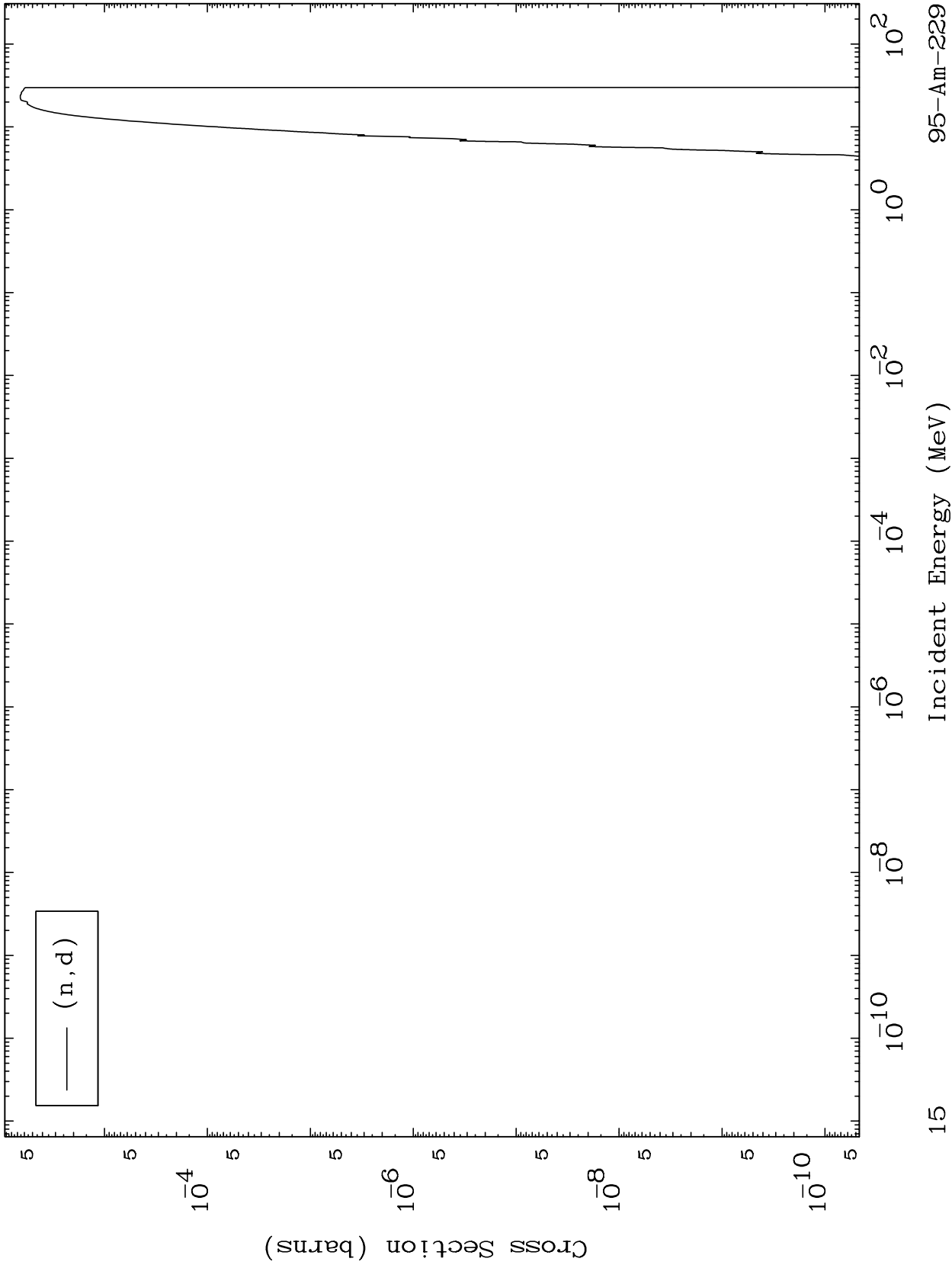
Incident Energy (MeV)

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(n,d) Levels  
293 Kelvin Cross Sections

95-Am-229



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Incident Energy (MeV)

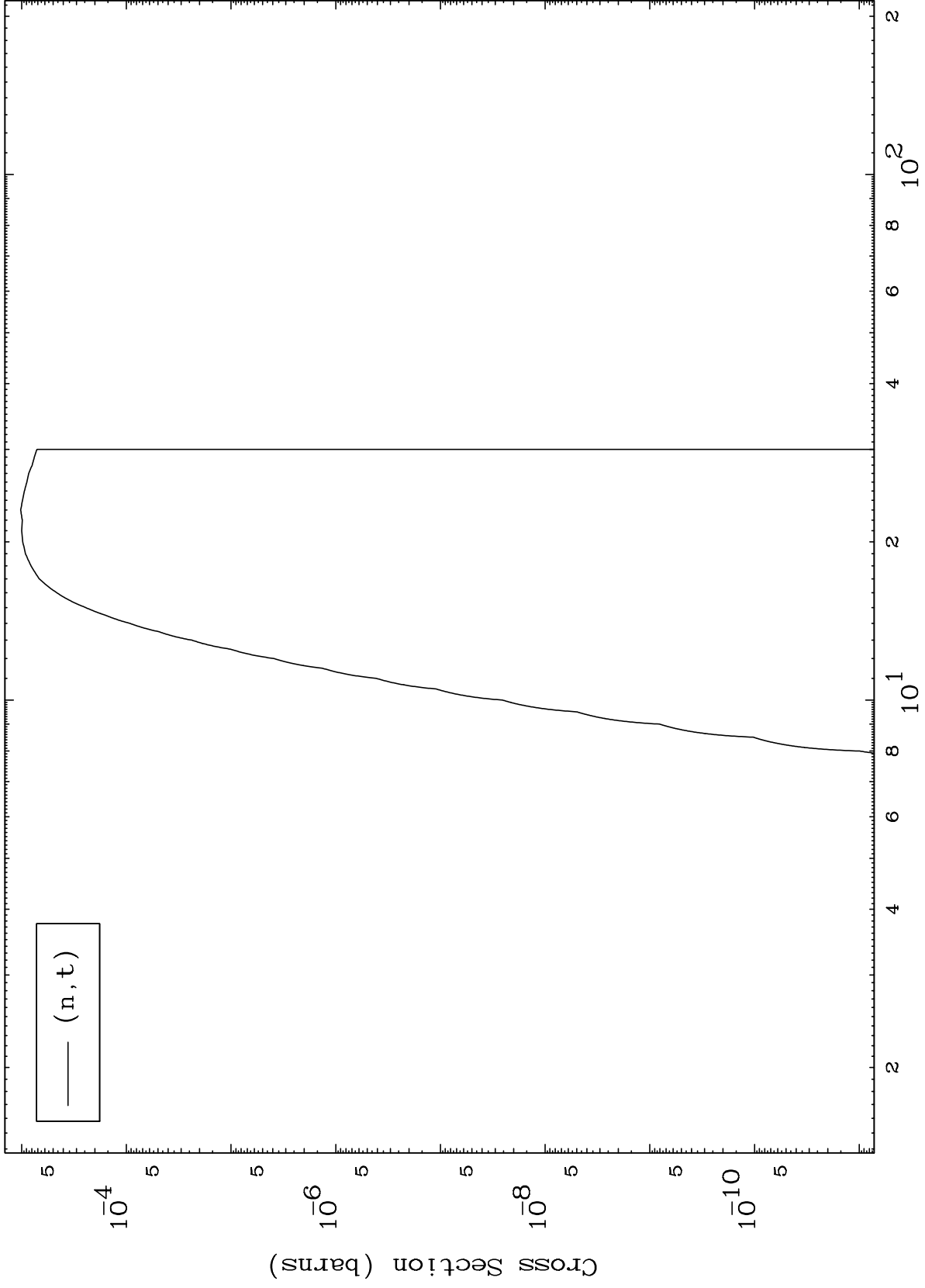
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(n,t) Levels  
293 Kelvin Cross Sections

95-Am-229



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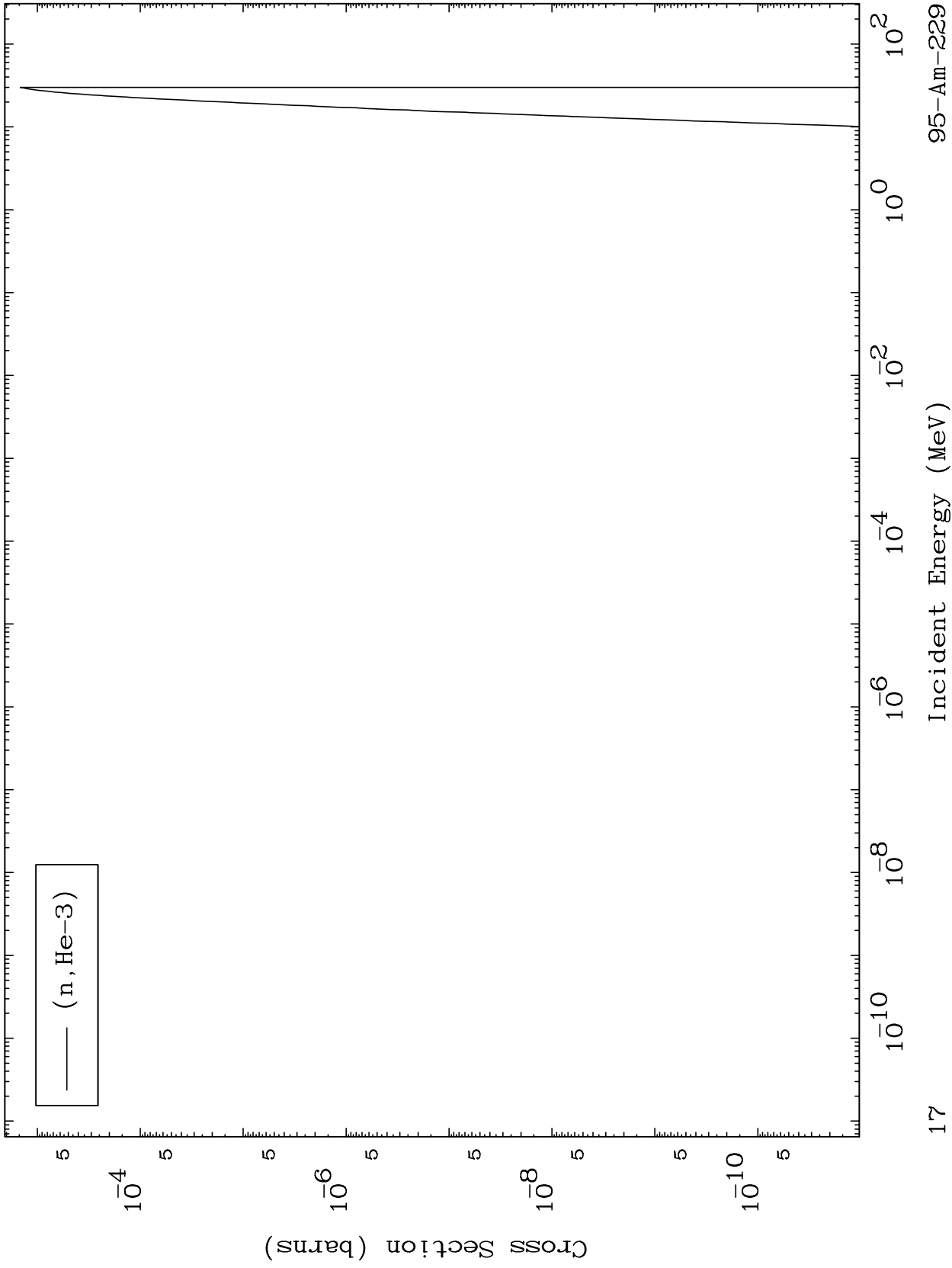
Incident Energy (MeV)

95-Am-229

MAT 9507

(n,He3) Levels  
293 Kelvin Cross Sections

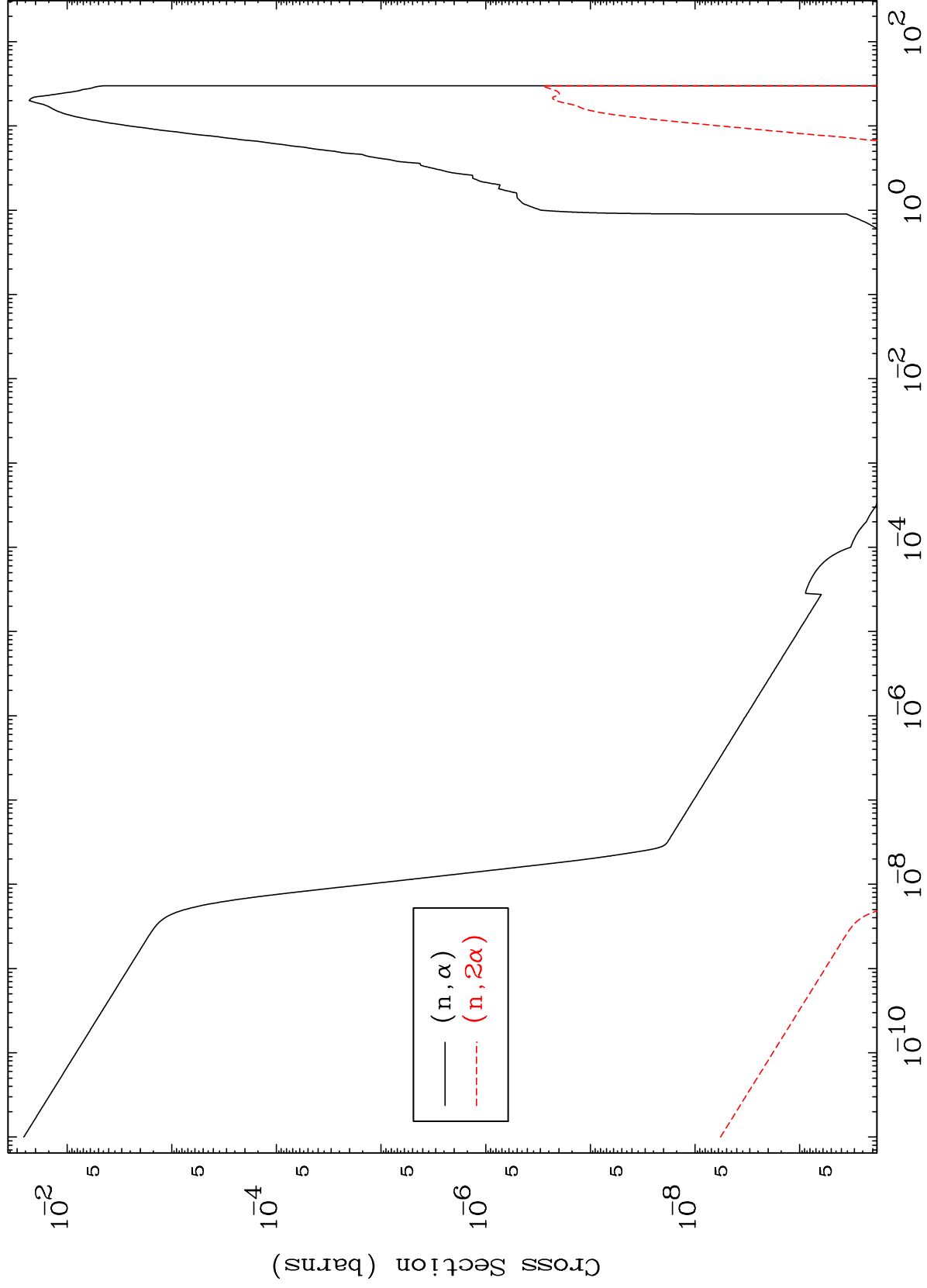
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

95-Am-229



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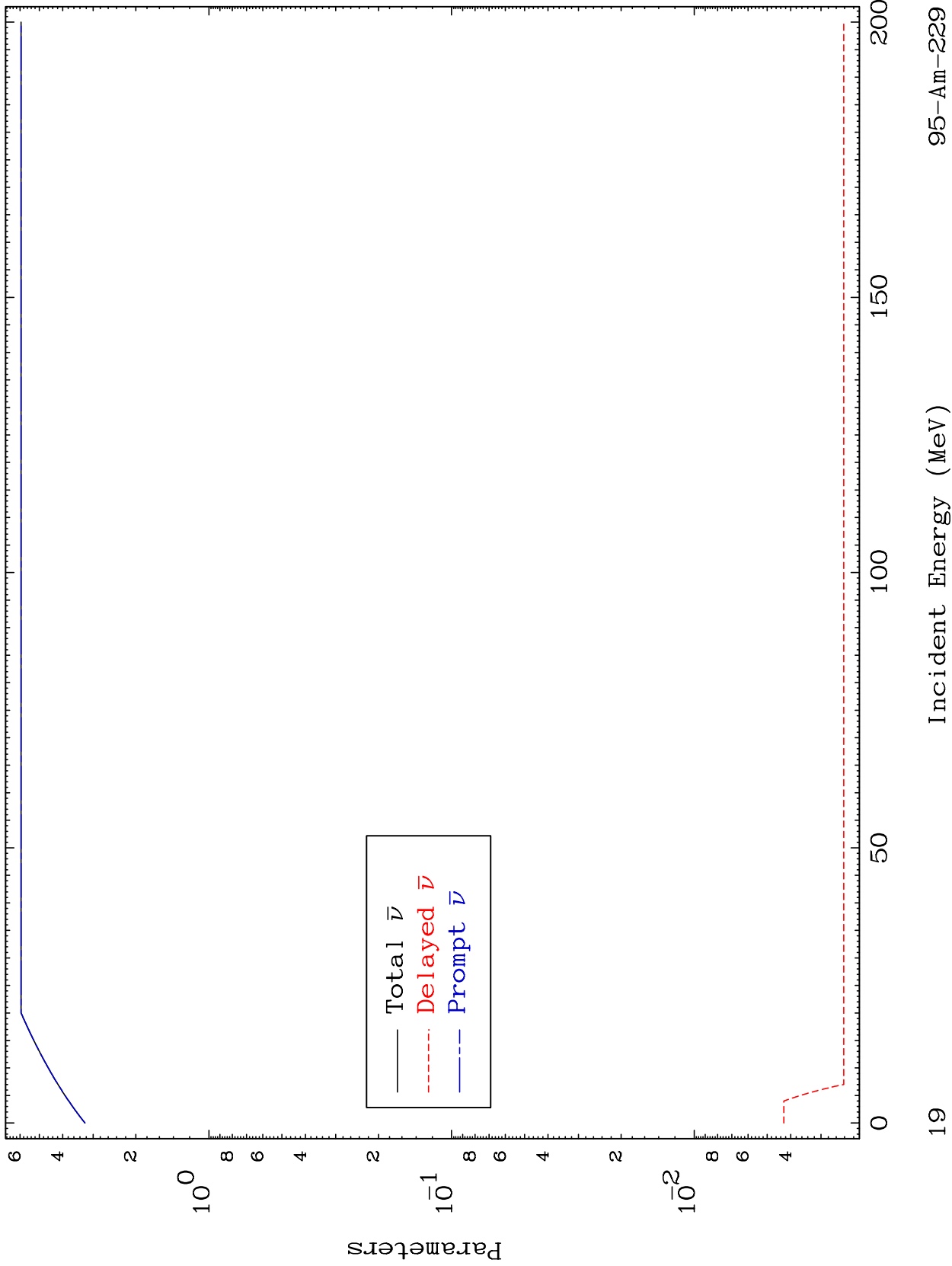
Incident Energy (MeV)

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Energy Release  
Parameters

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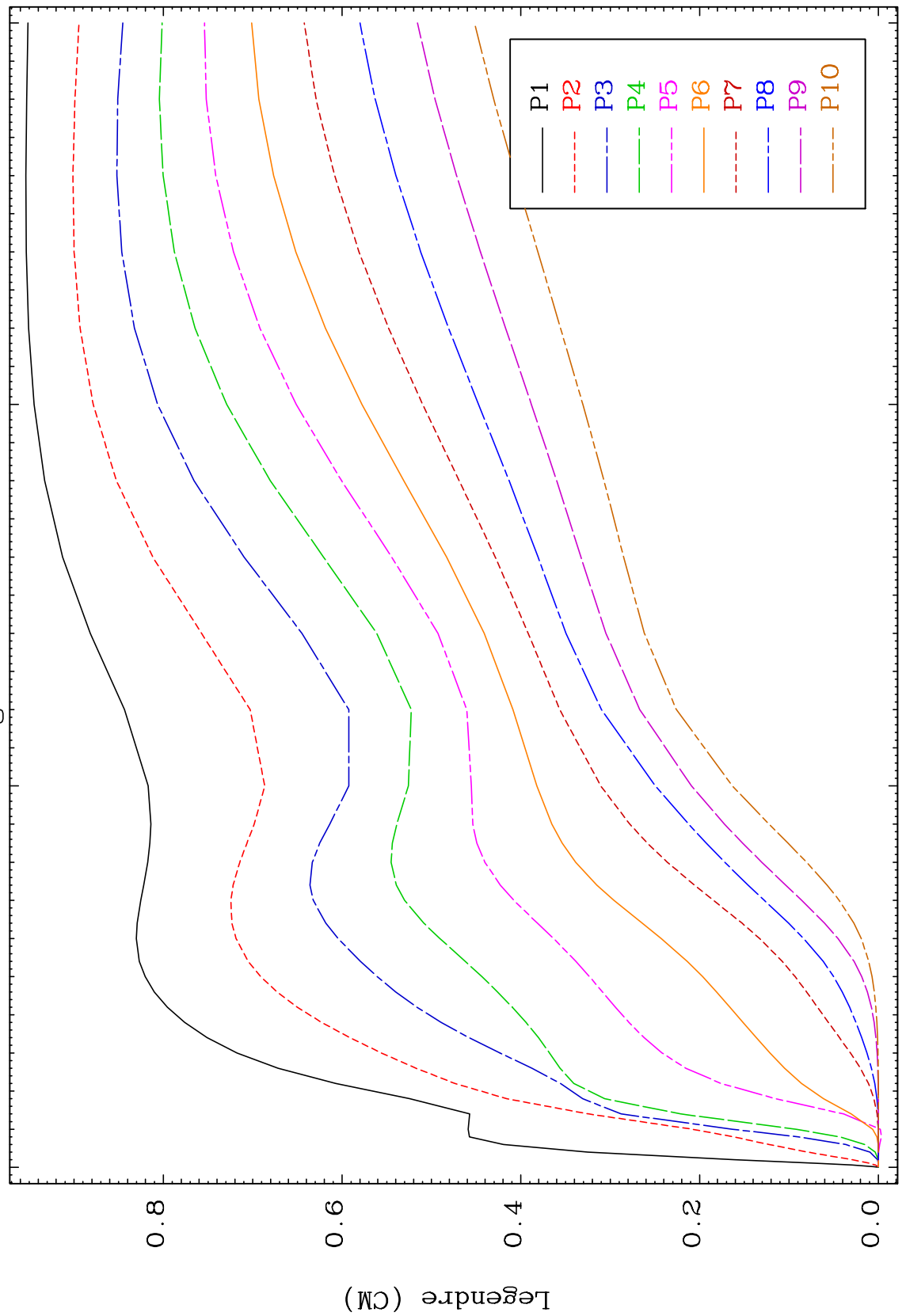
Incident Energy (MeV)

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Elastic Legendre Coefficients



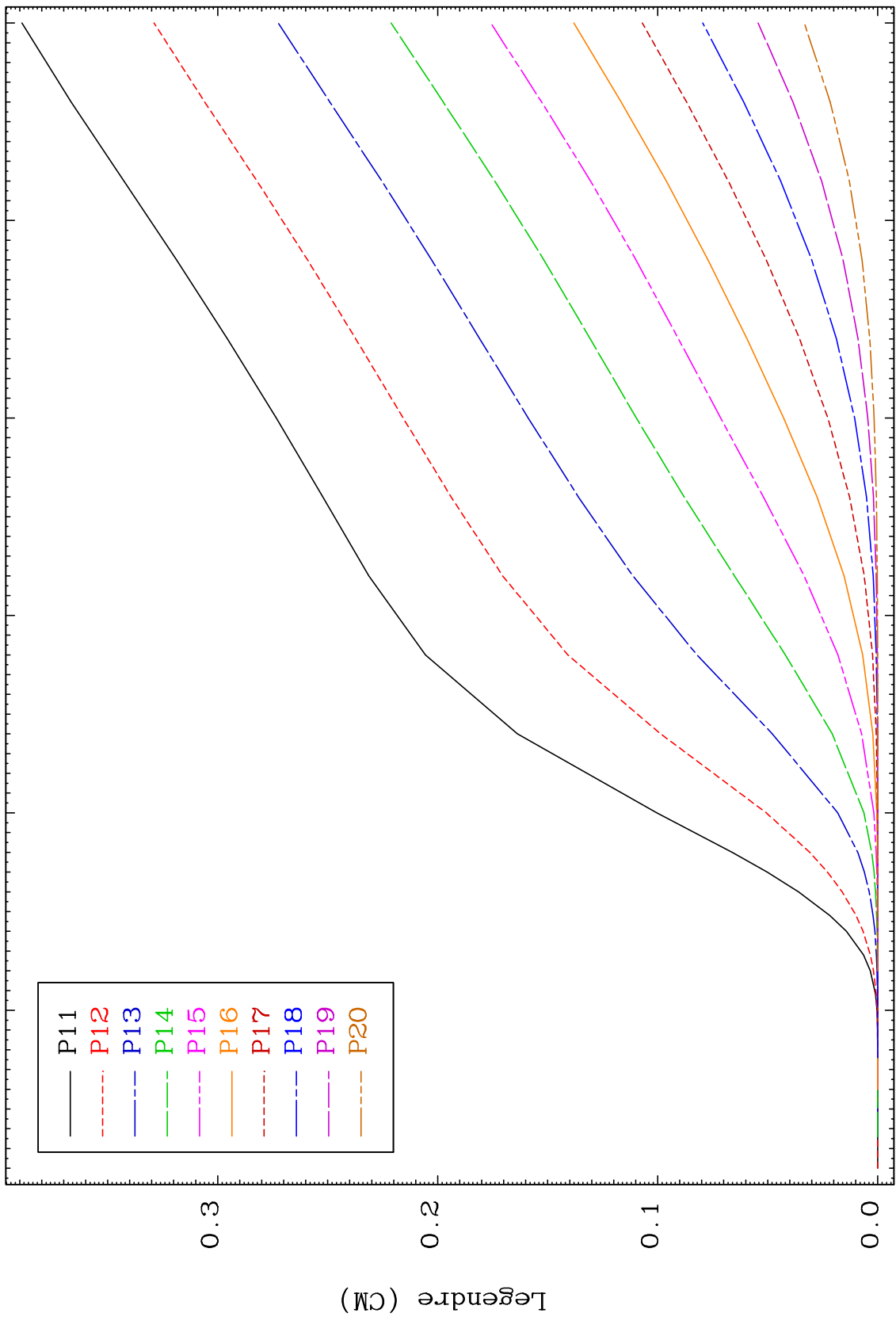
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Incident Energy (MeV)

20

30

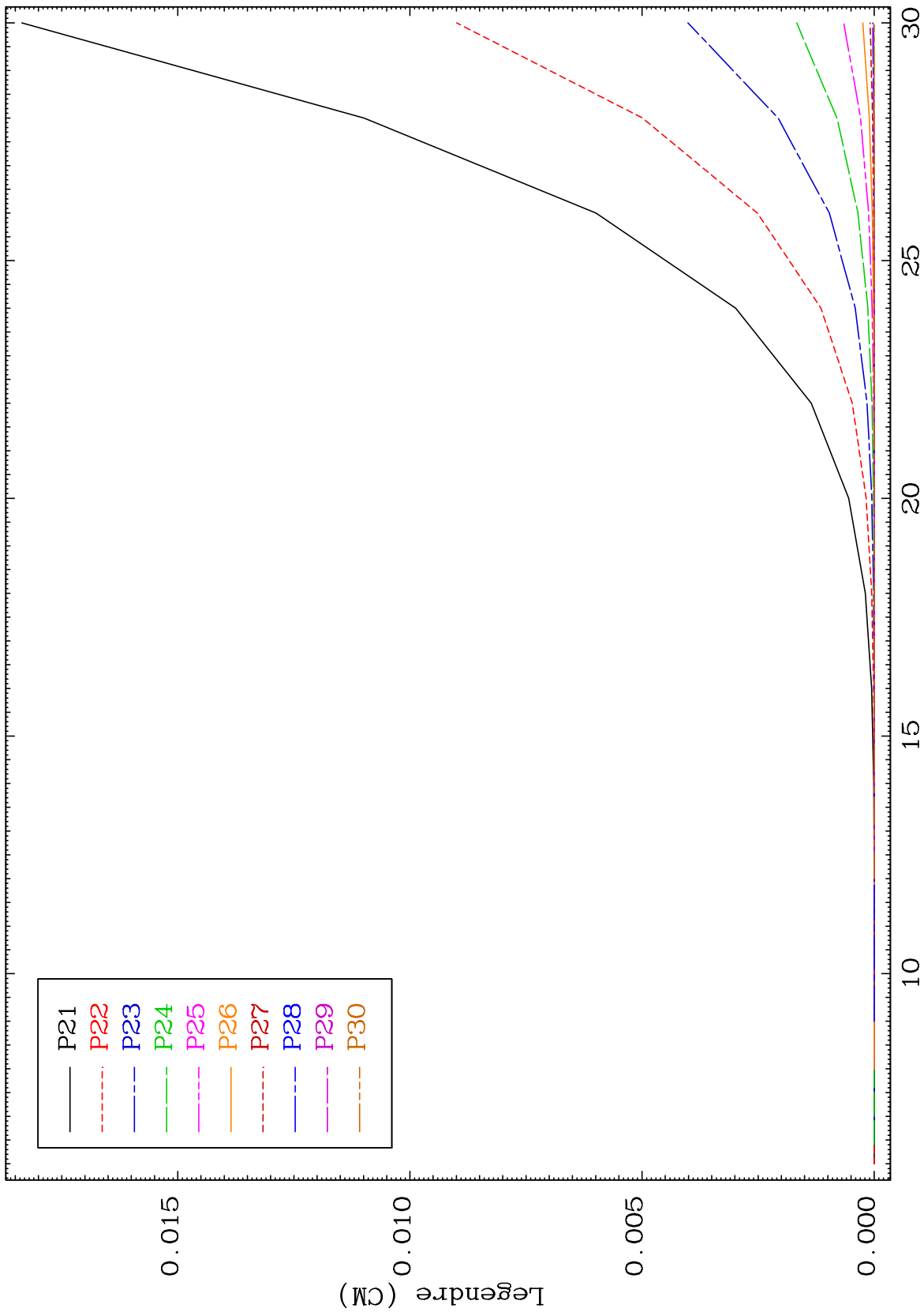
Elastic Legendre Coefficients



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### Elastic Legendre Coefficients

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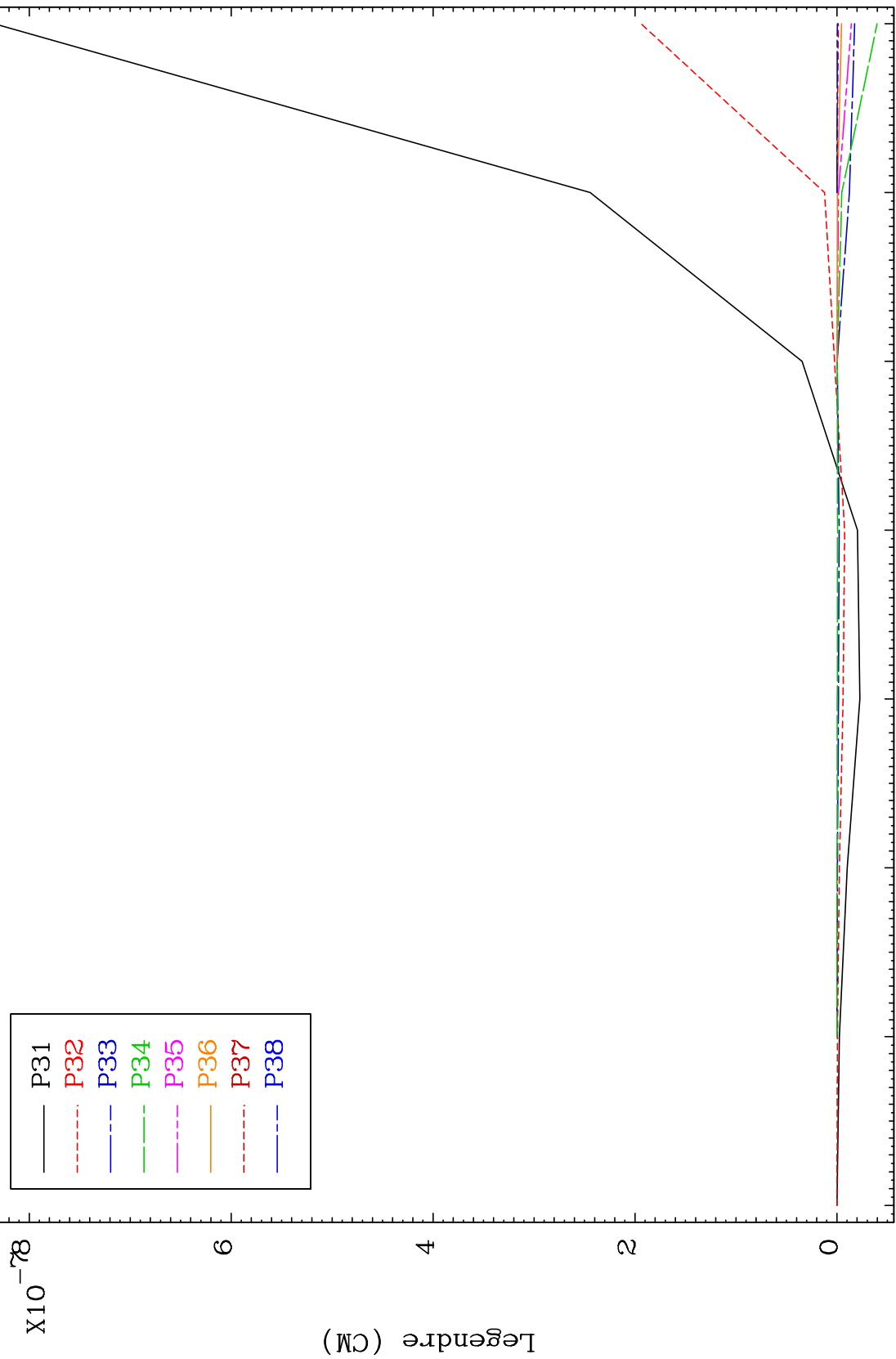
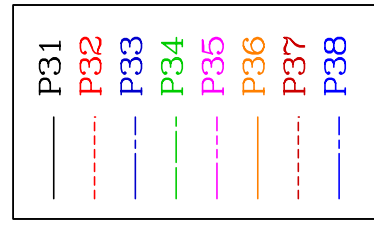
Incident Energy (MeV)

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Elastic Legendre Coefficients

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Incident Energy (MeV)

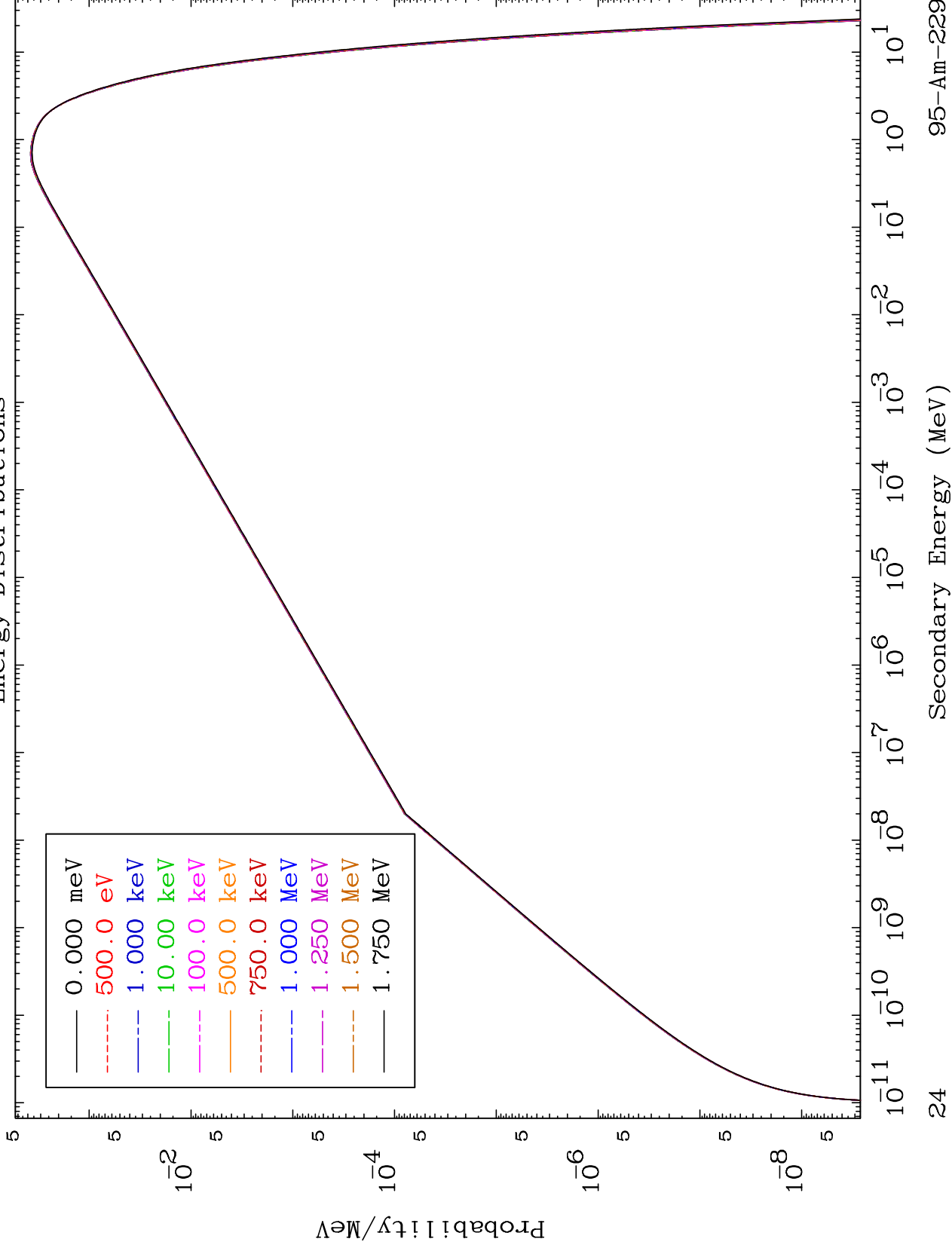
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### Fission Energy Distributions

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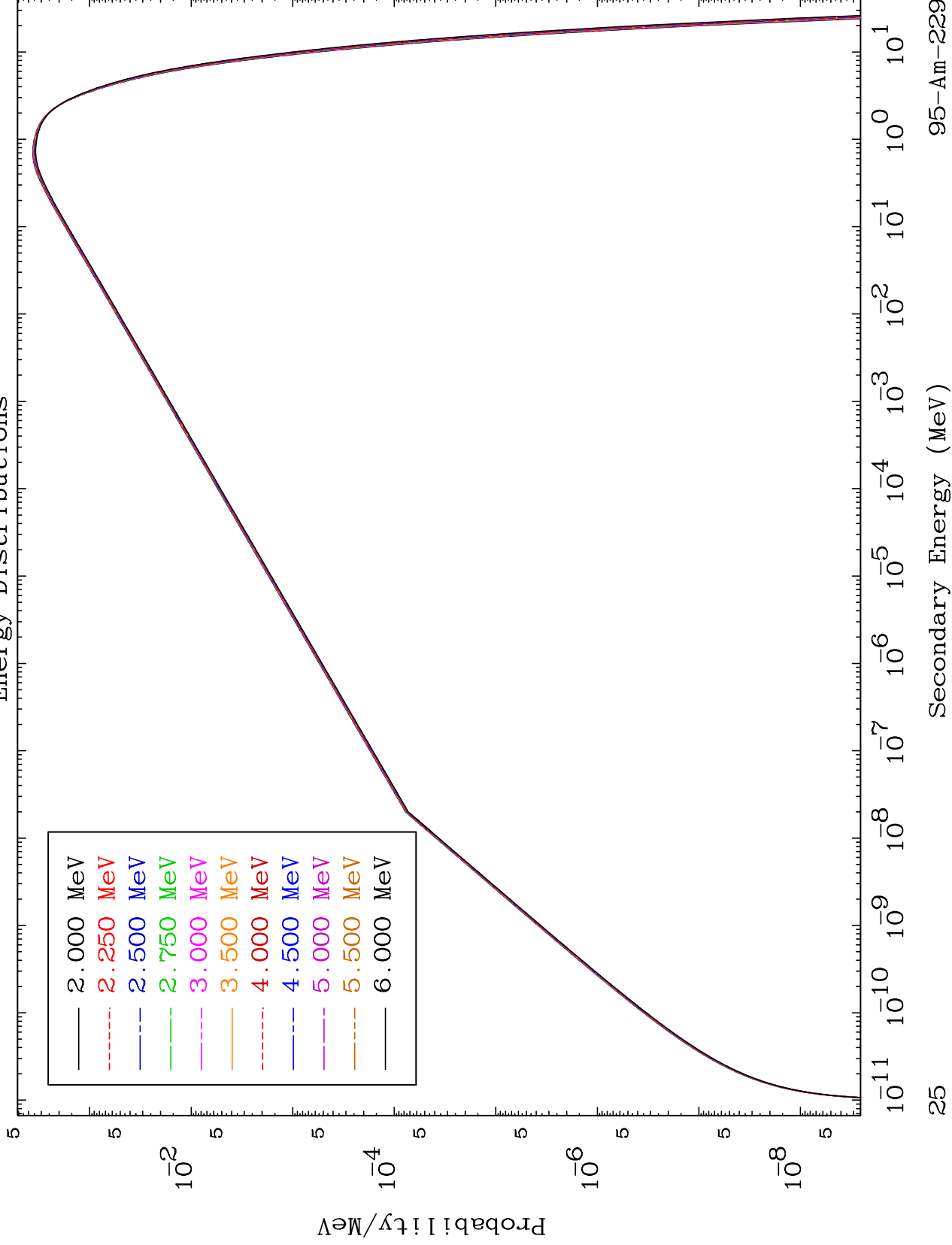
Secondary Energy (MeV)

24

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### Fission Energy Distributions

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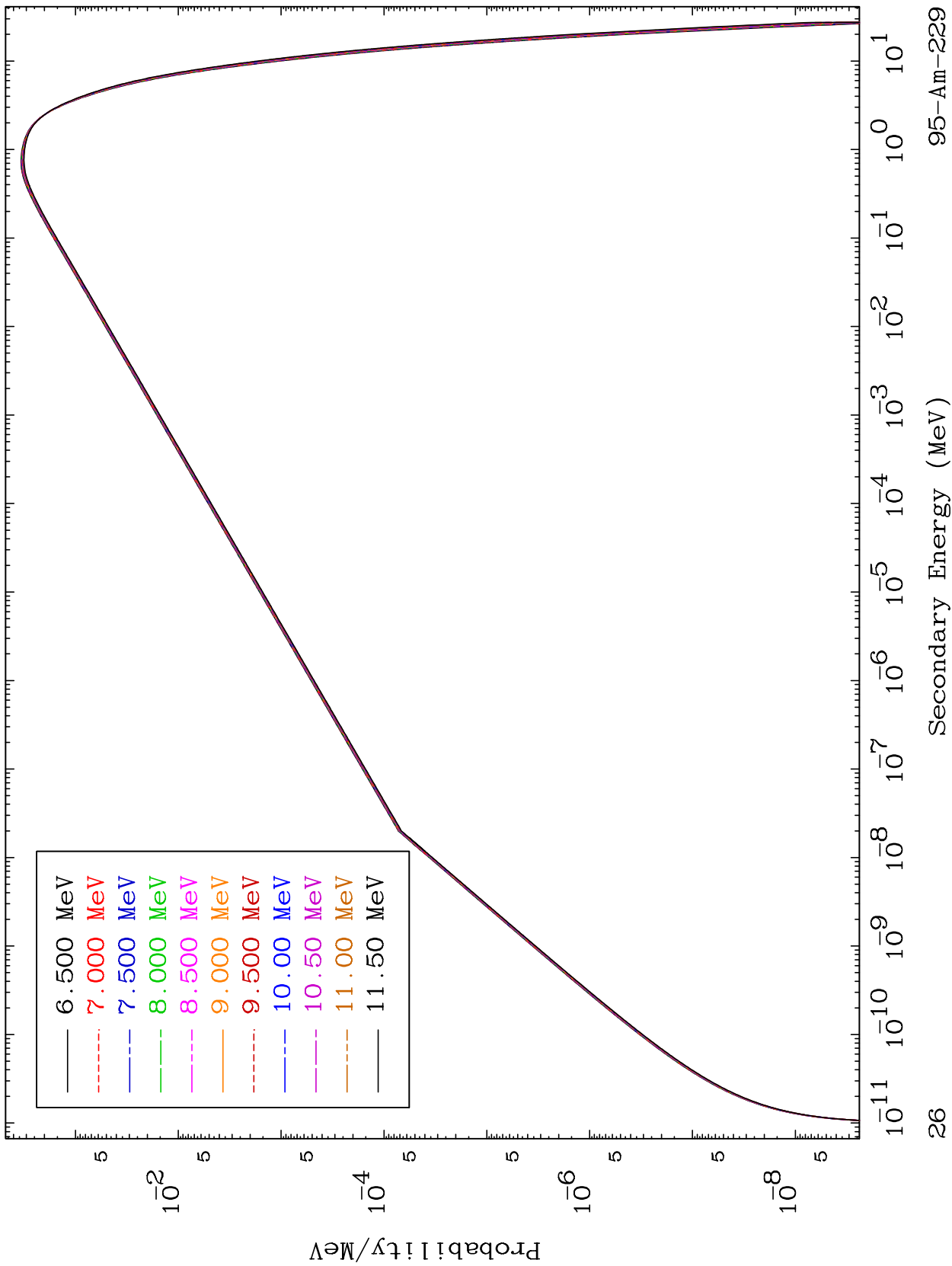
Secondary Energy (MeV)

25

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### Fission Energy Distributions

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95-Am-229

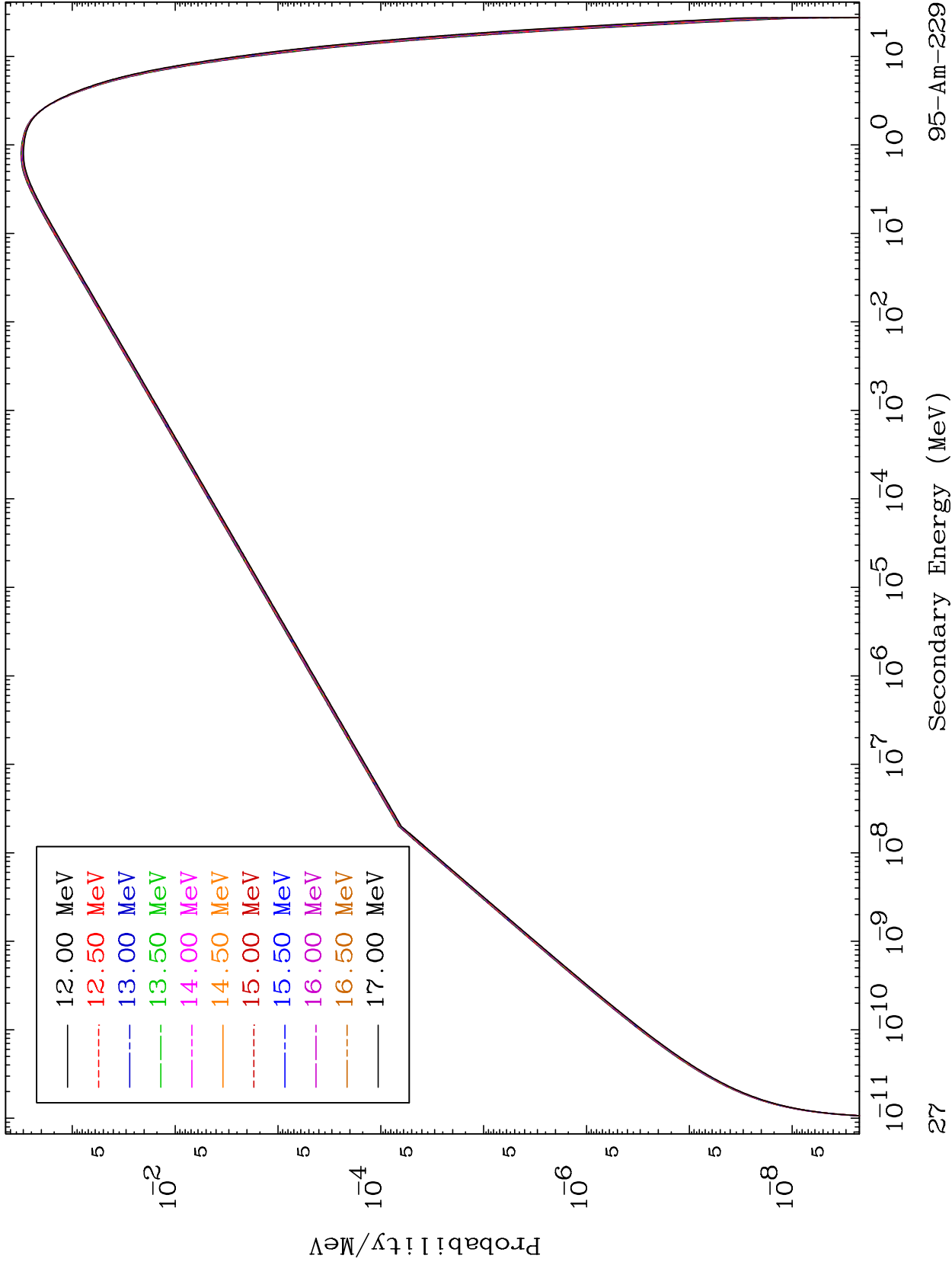
Secondary Energy (MeV)

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### Fission Energy Distributions

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95-Am-229

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### Fission Energy Distributions

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