

Program EVALPLOT
(Version 2021-1)

by

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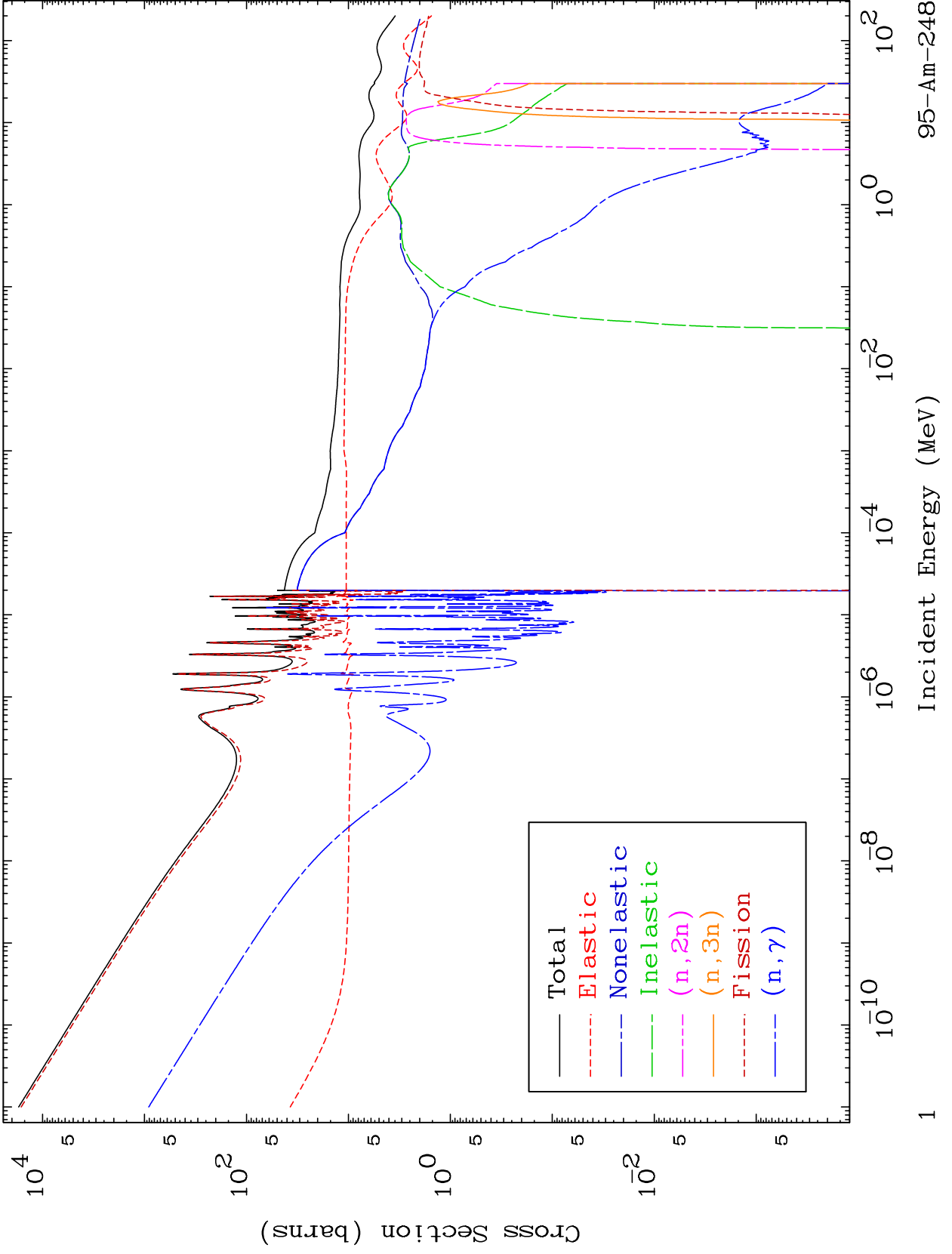
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Press Mouse Button to Start

MAT 9564

Neutron Major
293 Kelvin Cross Sections

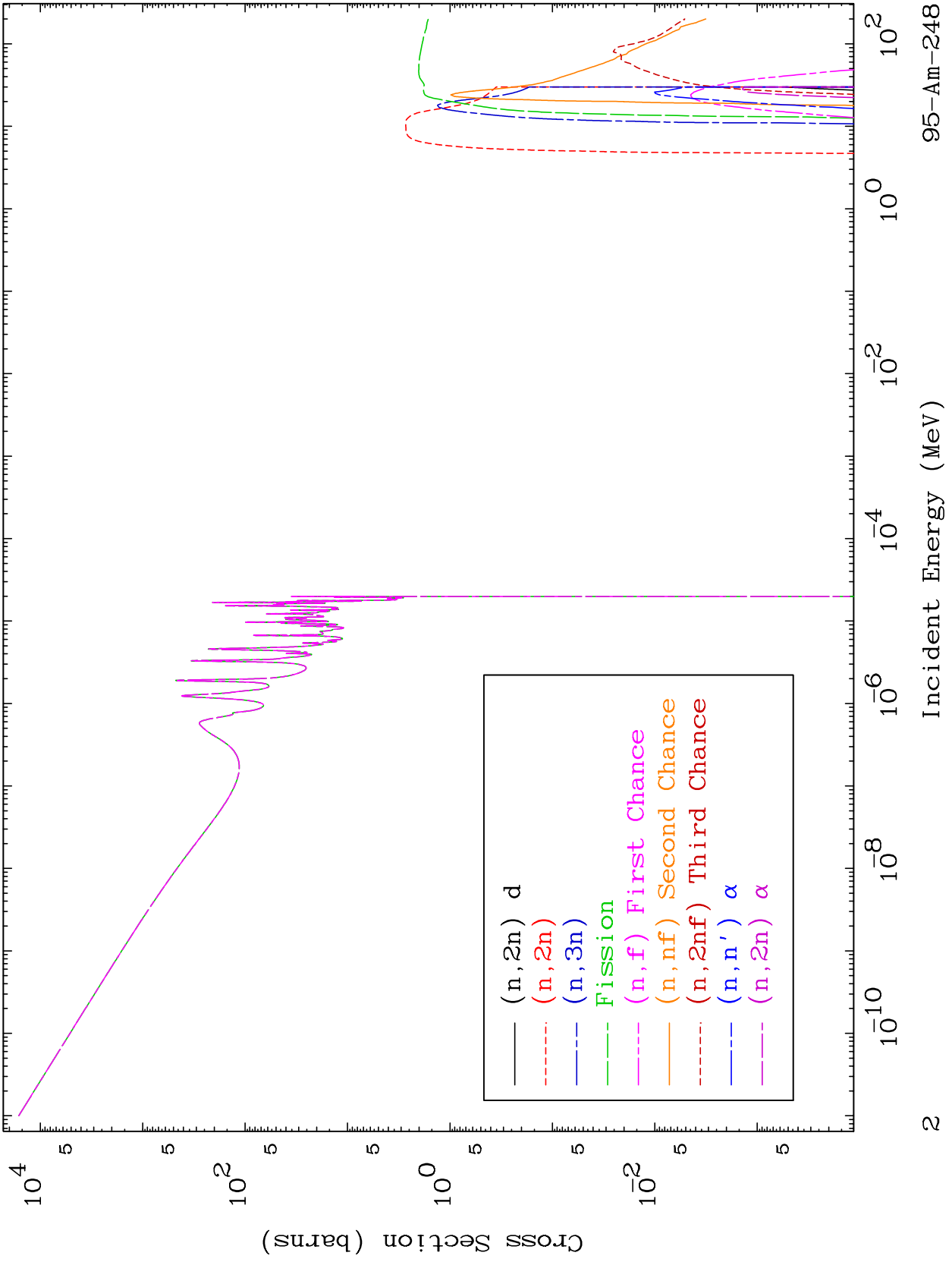
95-Am-248



MAT 9564

Neutron Absorption
293 Kelvin Cross Sections

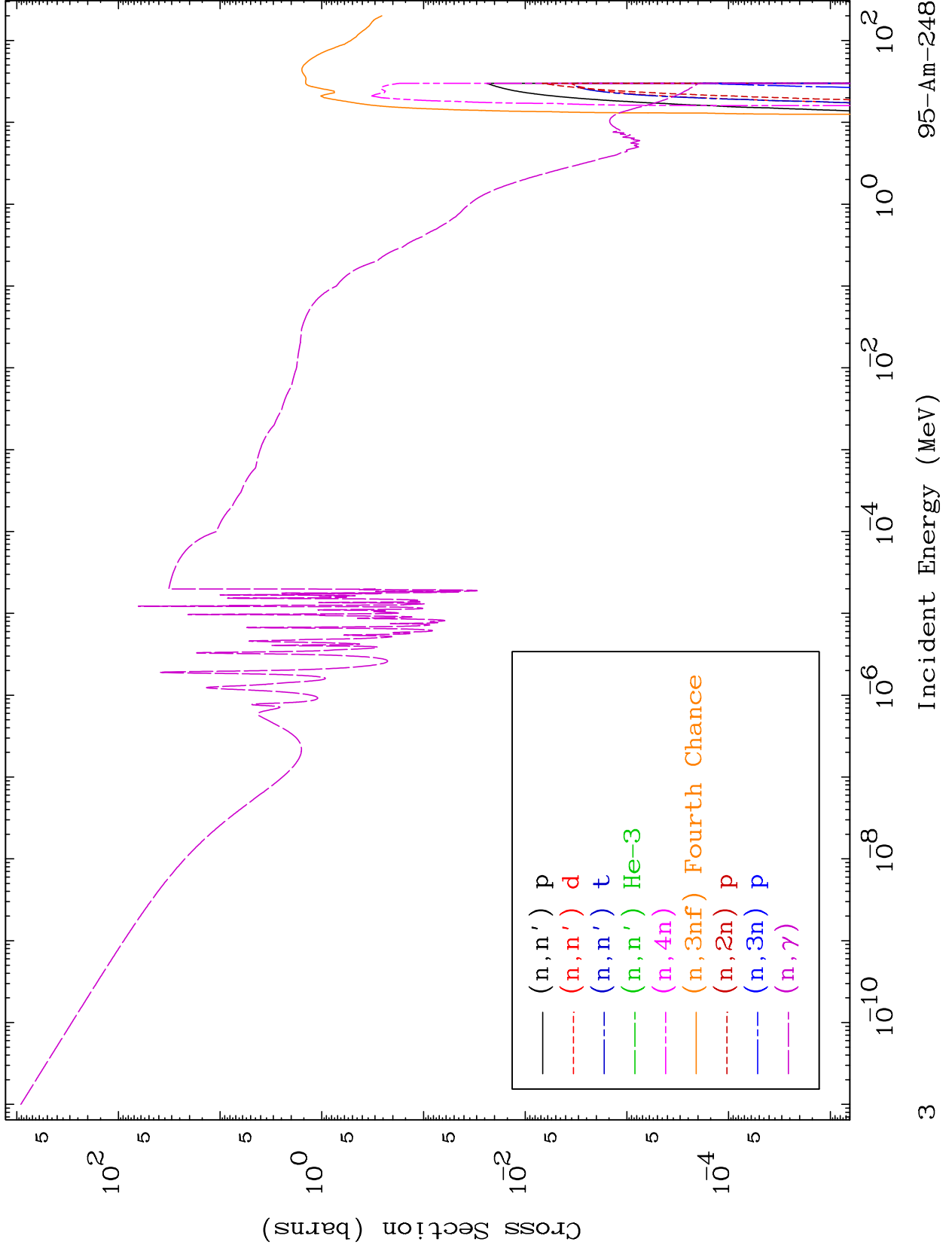
95-Am-248



MAT 9564

Neutron Absorption
293 Kelvin Cross Sections

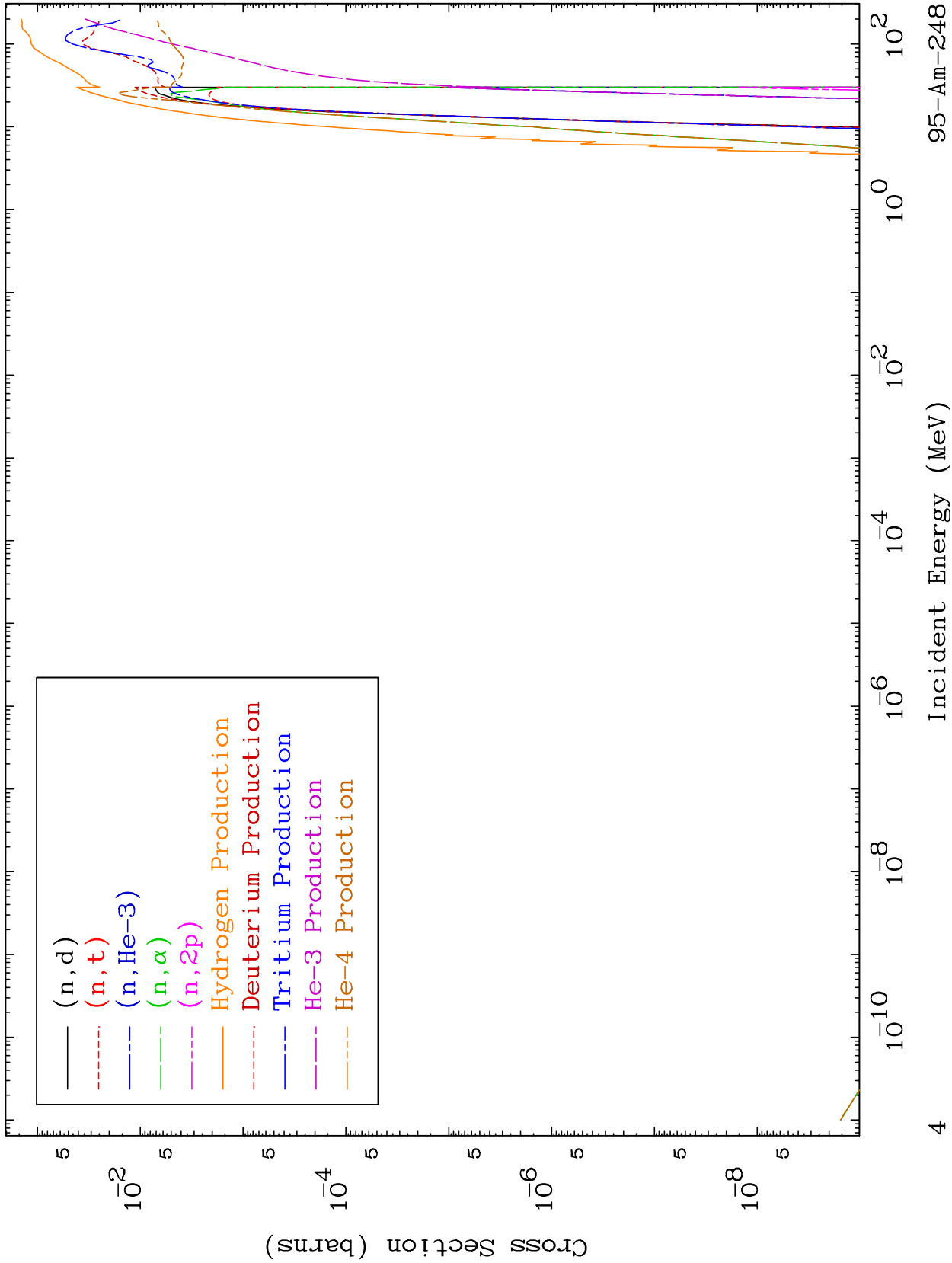
95-Am-248



MAT 9564

Neutron Absorption
293 Kelvin Cross Sections

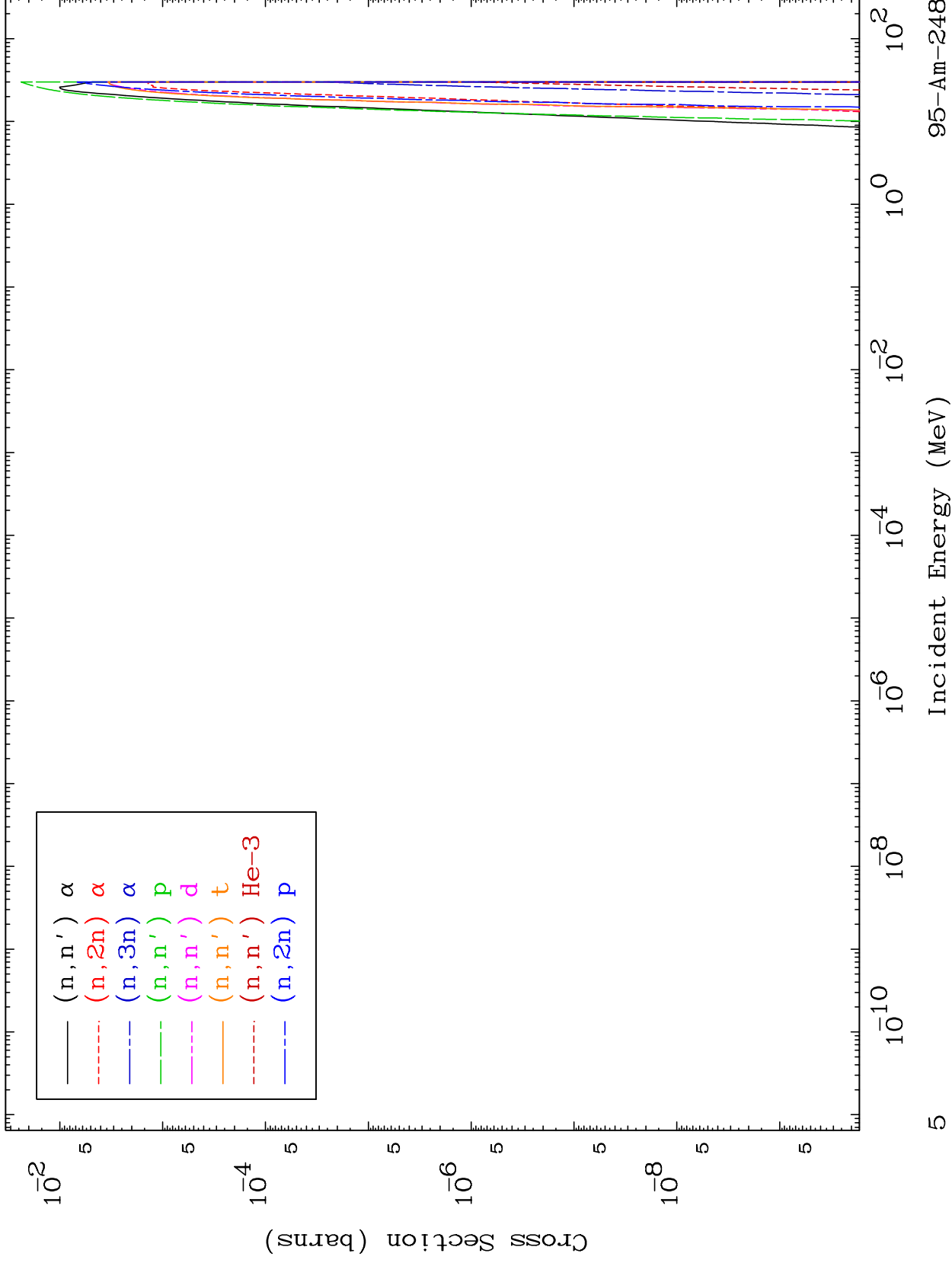
95-Am-248



MAT 9564

Charged Particle
293 Kelvin Cross Sections

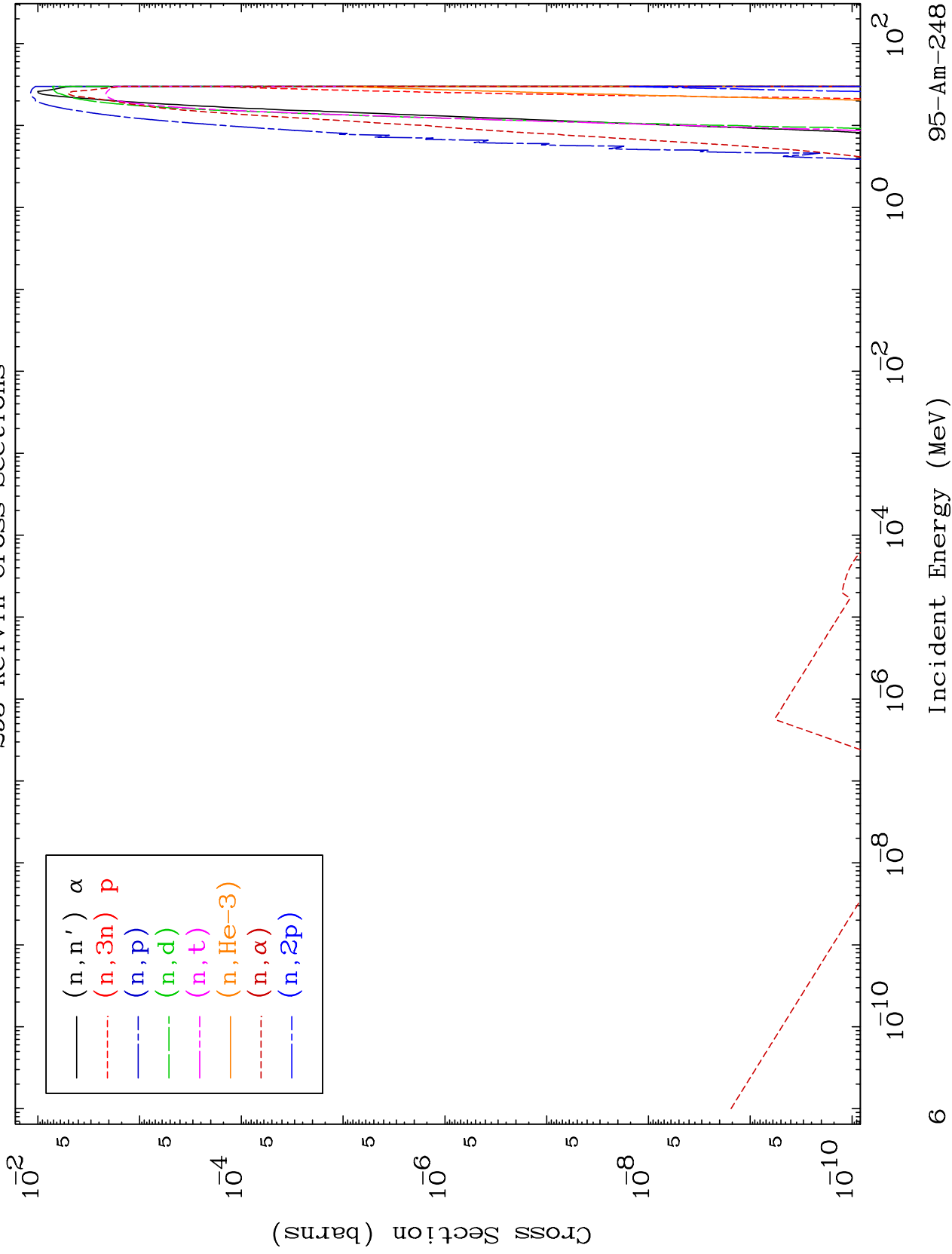
95-Am-248



MAT 9564

Charged Particle
293 Kelvin Cross Sections

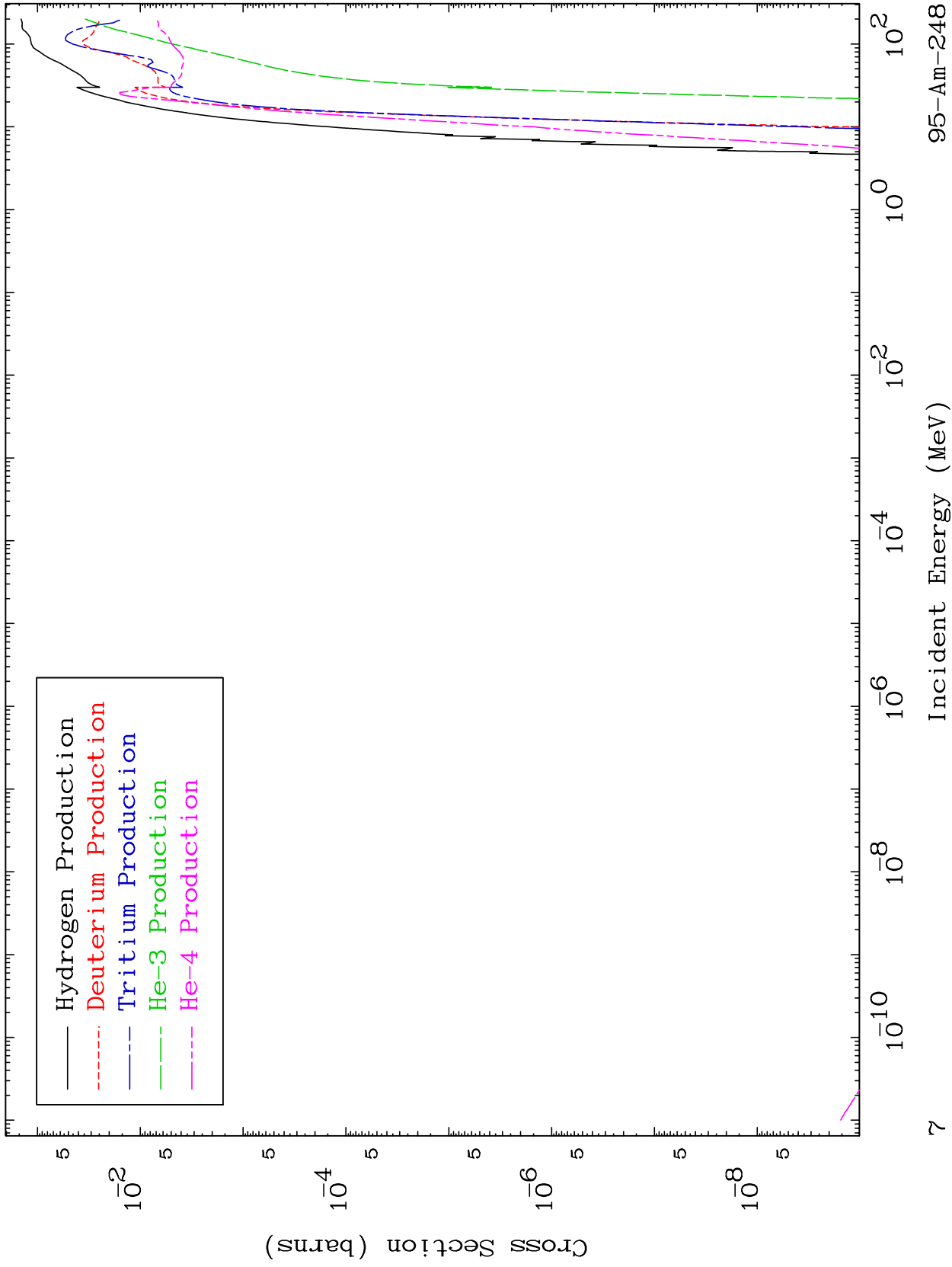
95-Am-248



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Particle Production
293 Kelvin Cross Sections

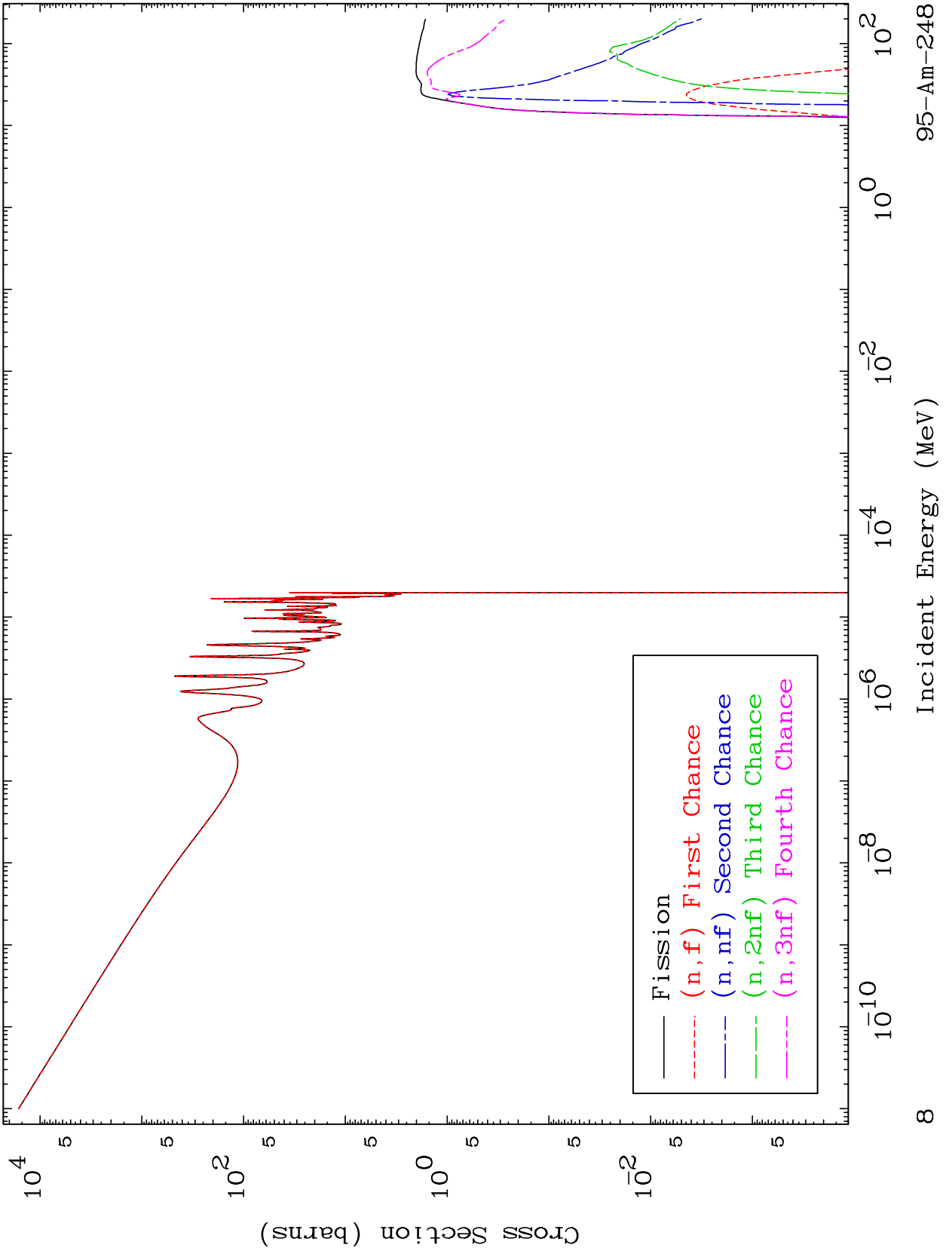
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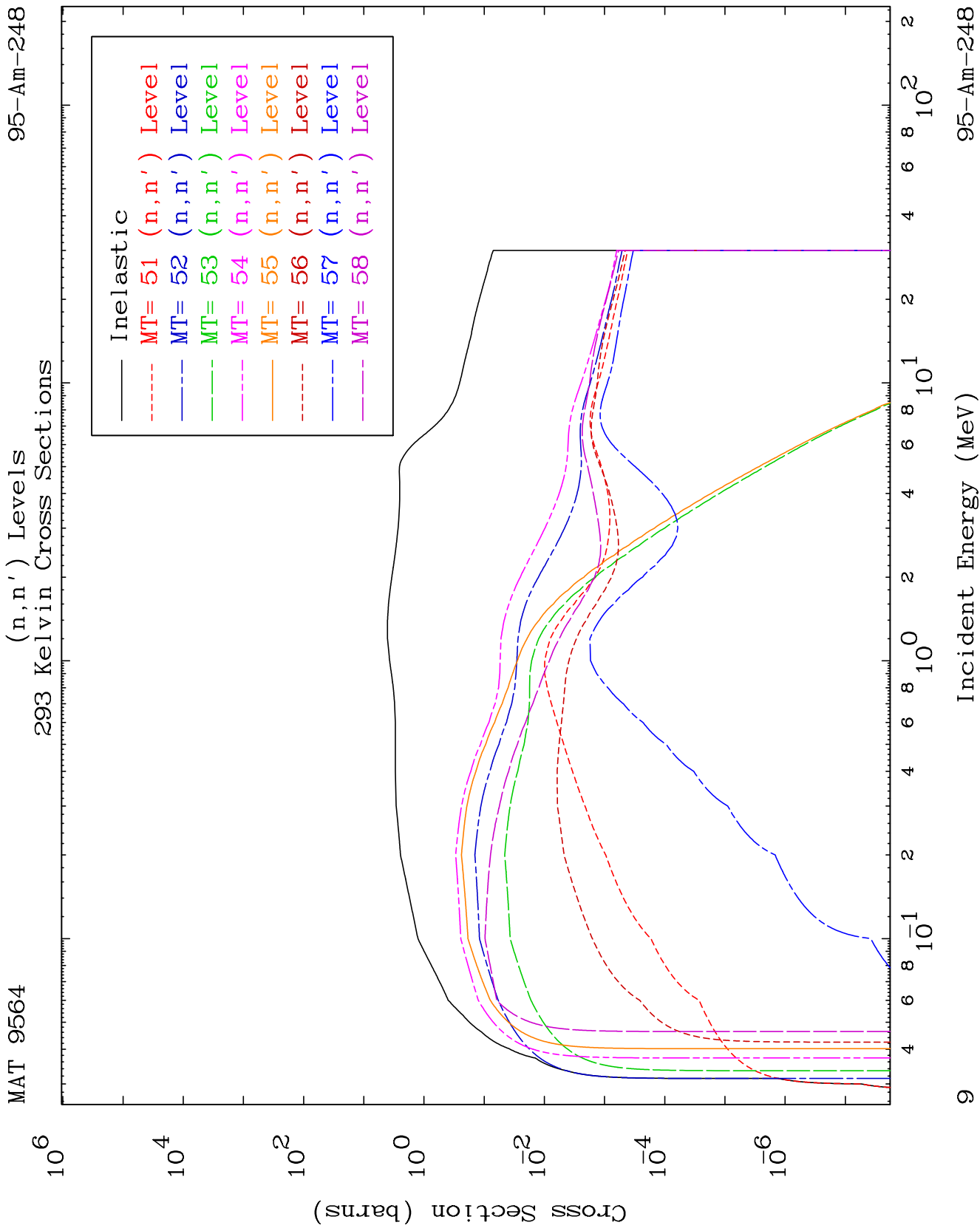


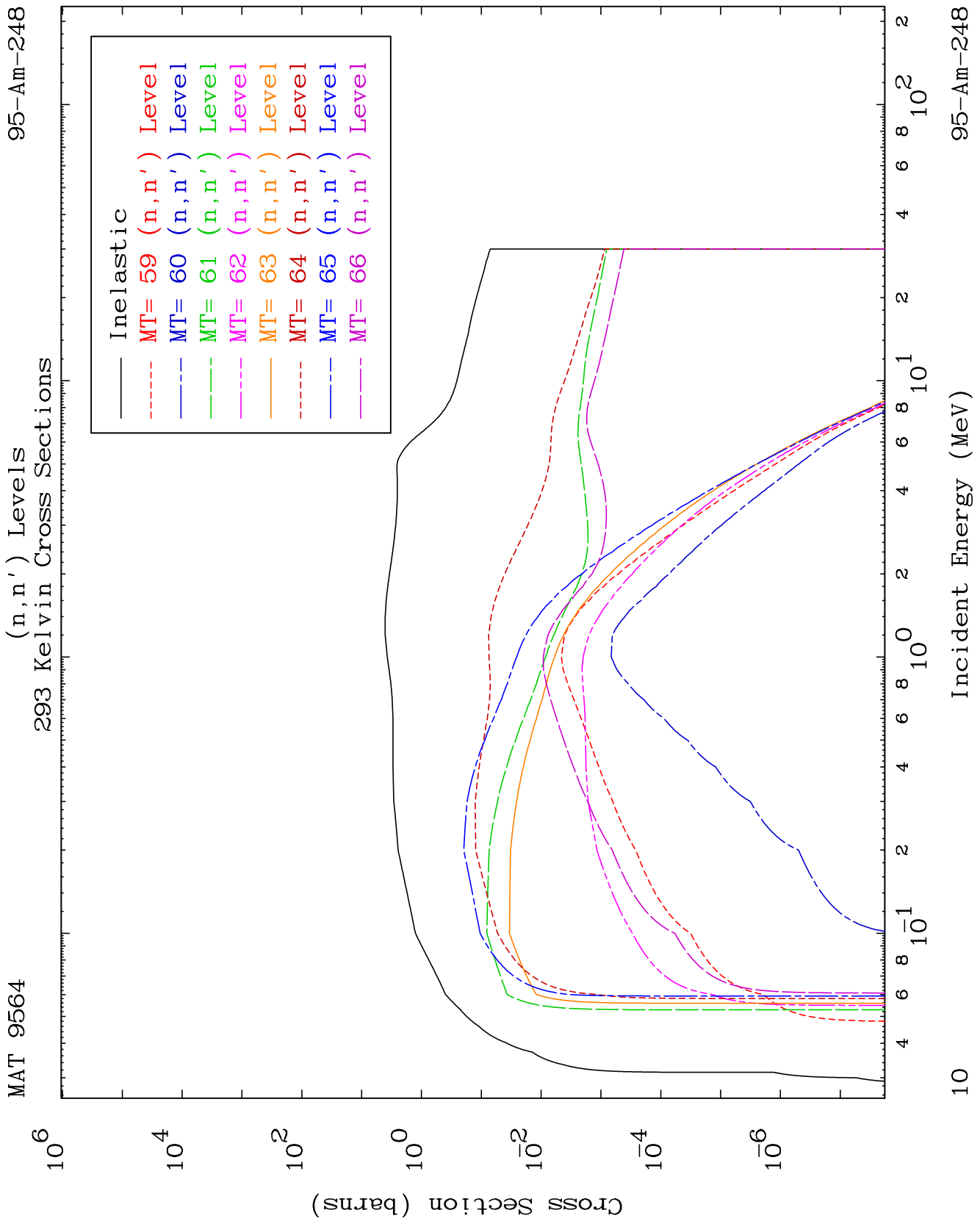
MAT 9564

Fission
293 Kelvin Cross Sections

95-Am-248







MAT 9564

(n,n') Levels

95-Am-248

293 Kelvin Cross Sections

10^6

10^4

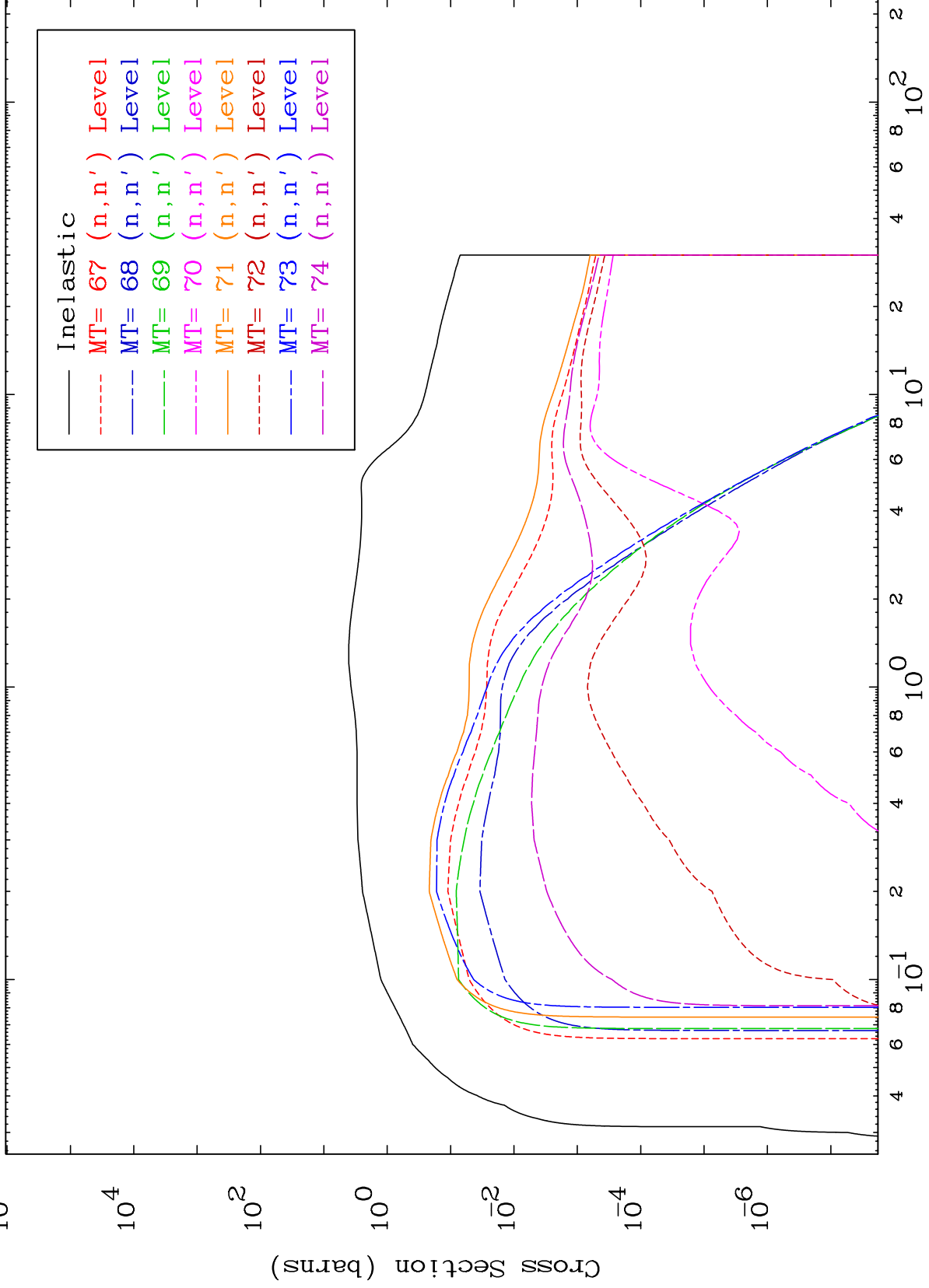
10^2

10^0

10^{-2}

10^{-4}

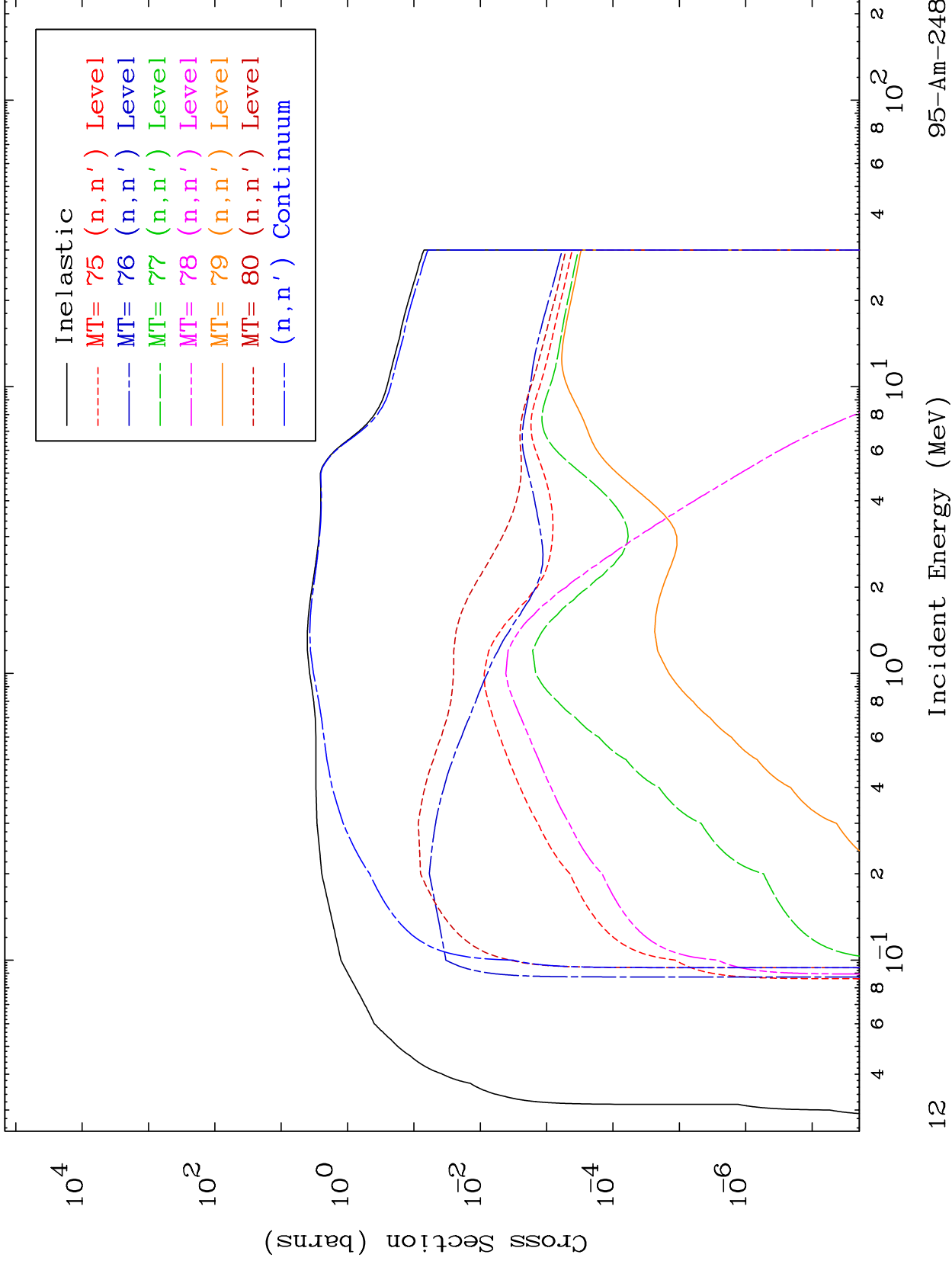
10^{-6}



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(n,n') Levels
293 Kelvin Cross Sections

95-Am-248



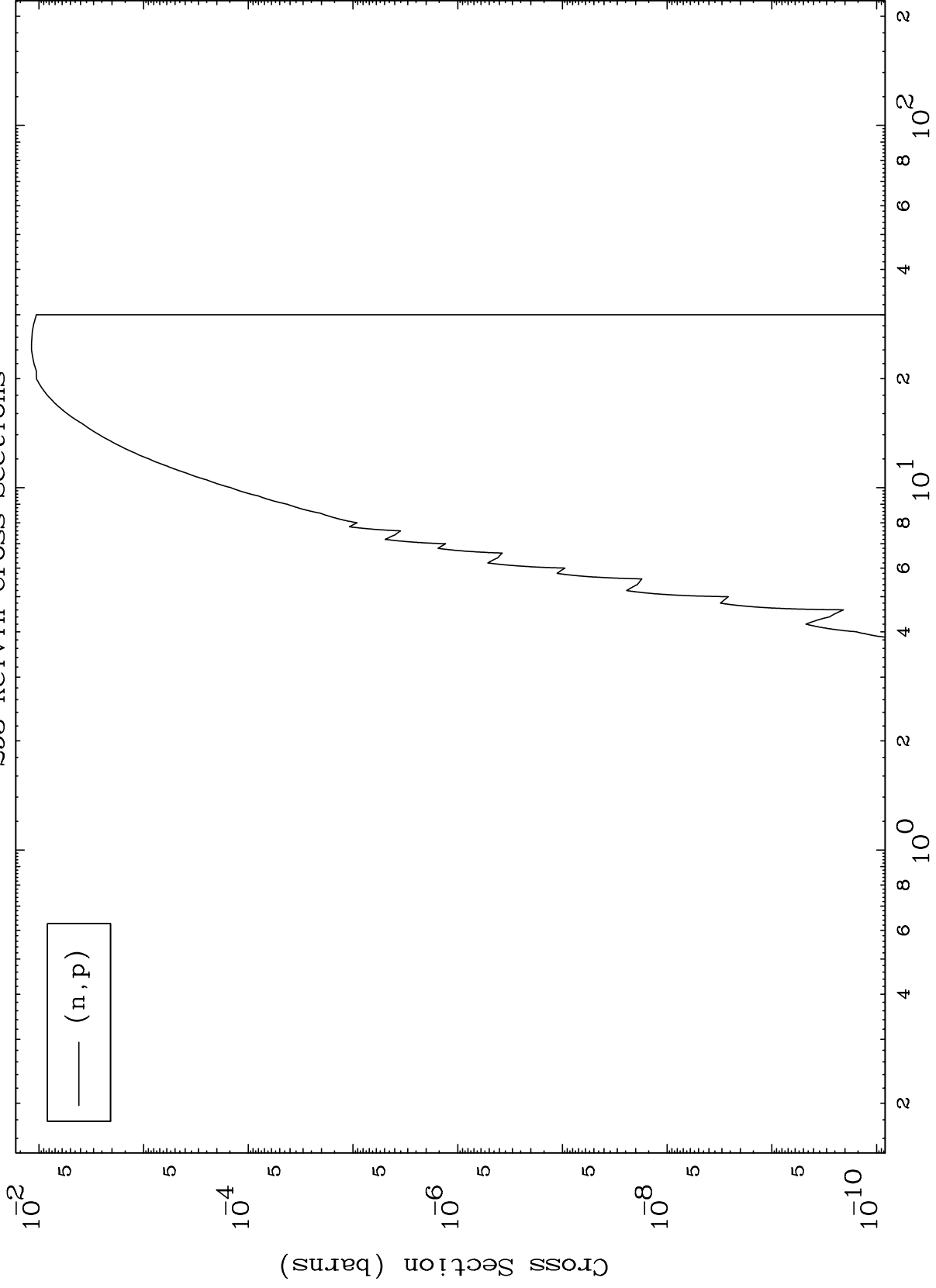
12

95-Am-248

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(n,p) Levels
293 Kelvin Cross Sections

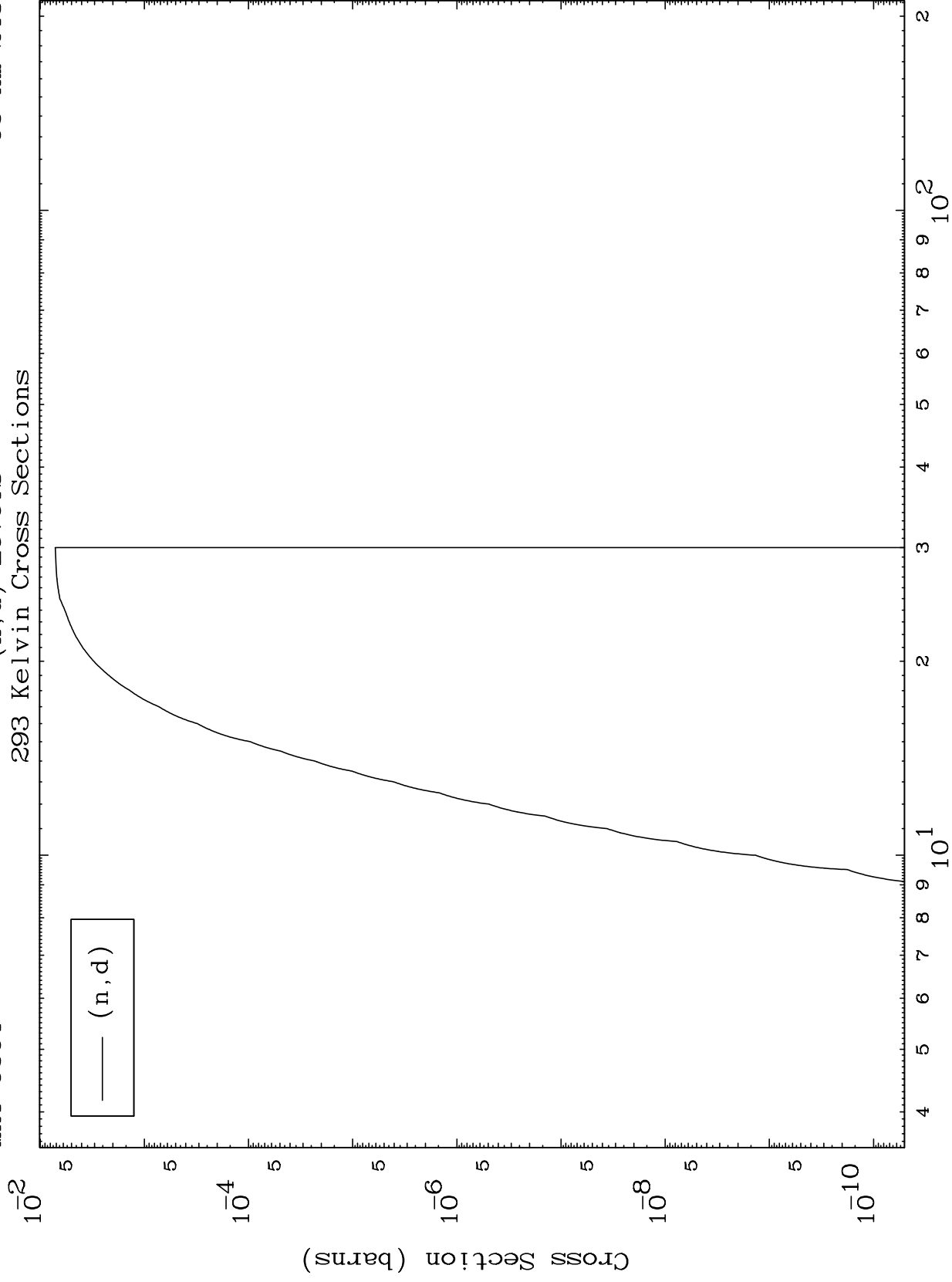
95-Am-248



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(n,d) Levels
293 Kelvin Cross Sections

95-Am-248



14

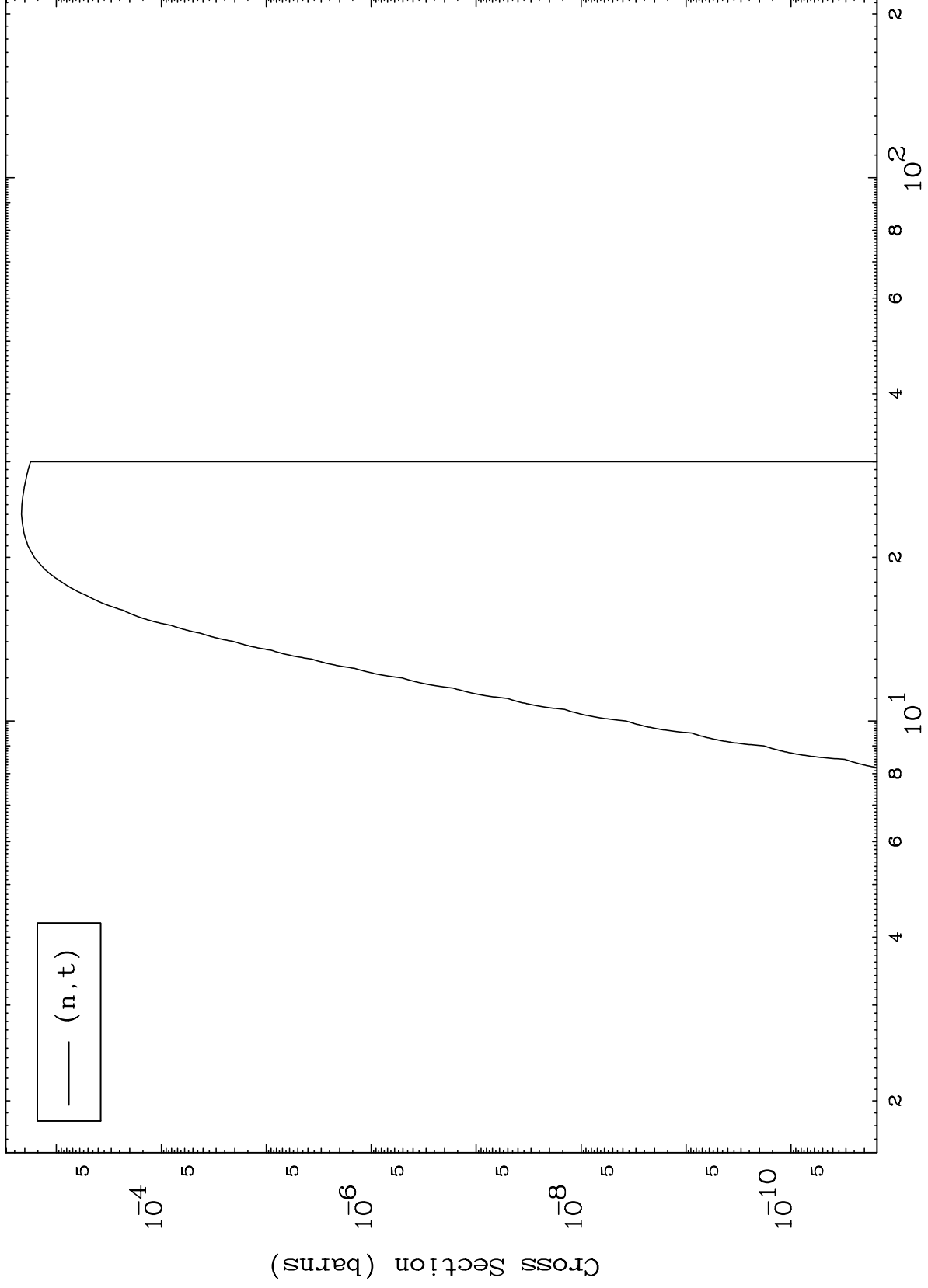
Incident Energy (MeV)

95-Am-248

MAT 9564

(n,t) Levels
293 Kelvin Cross Sections

95-Am-248



(n,t)

15

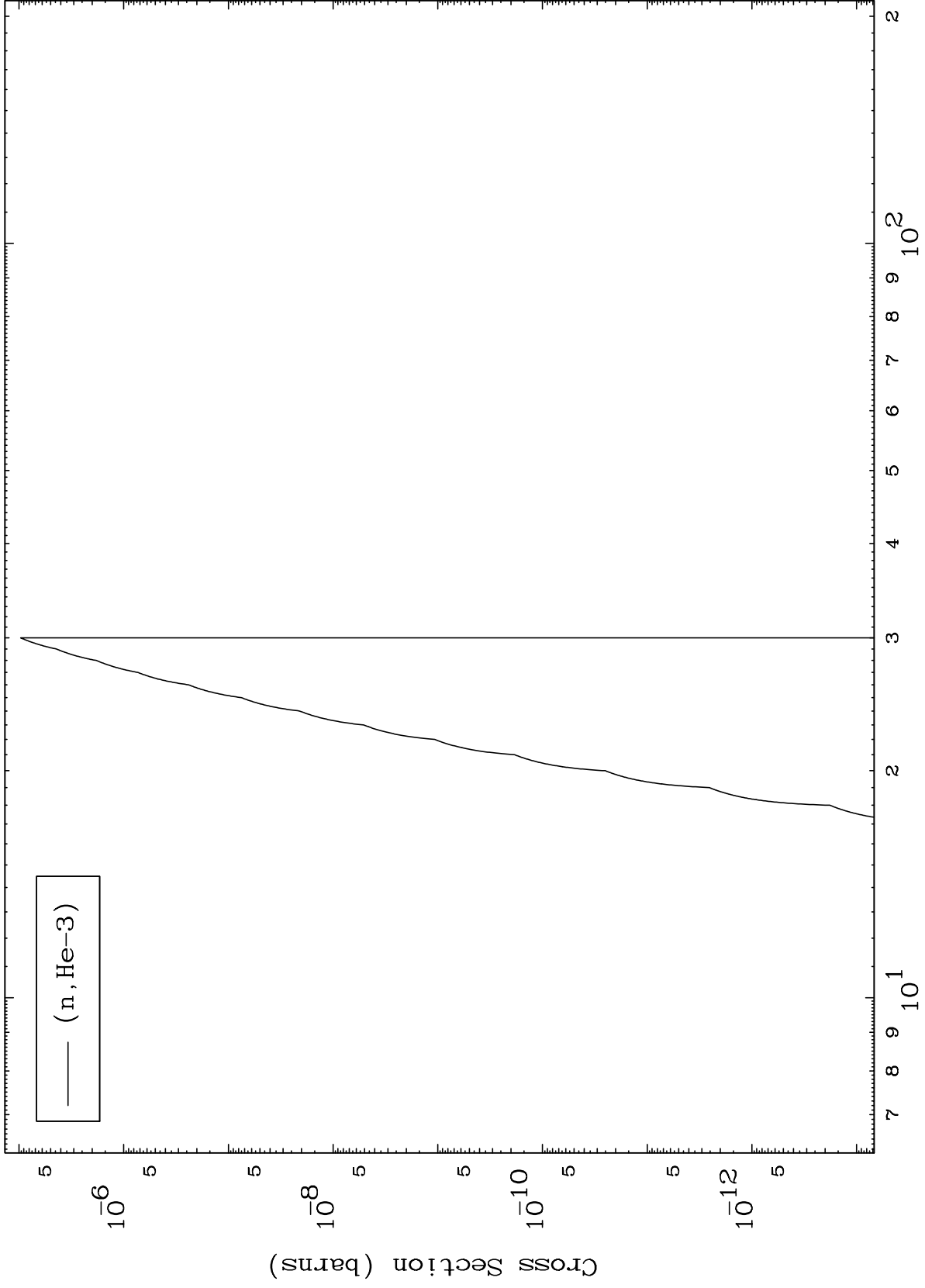
Incident Energy (MeV)

95-Am-248

MAT 9564

(n,He3) Levels
293 Kelvin Cross Sections

95-Am-248



16

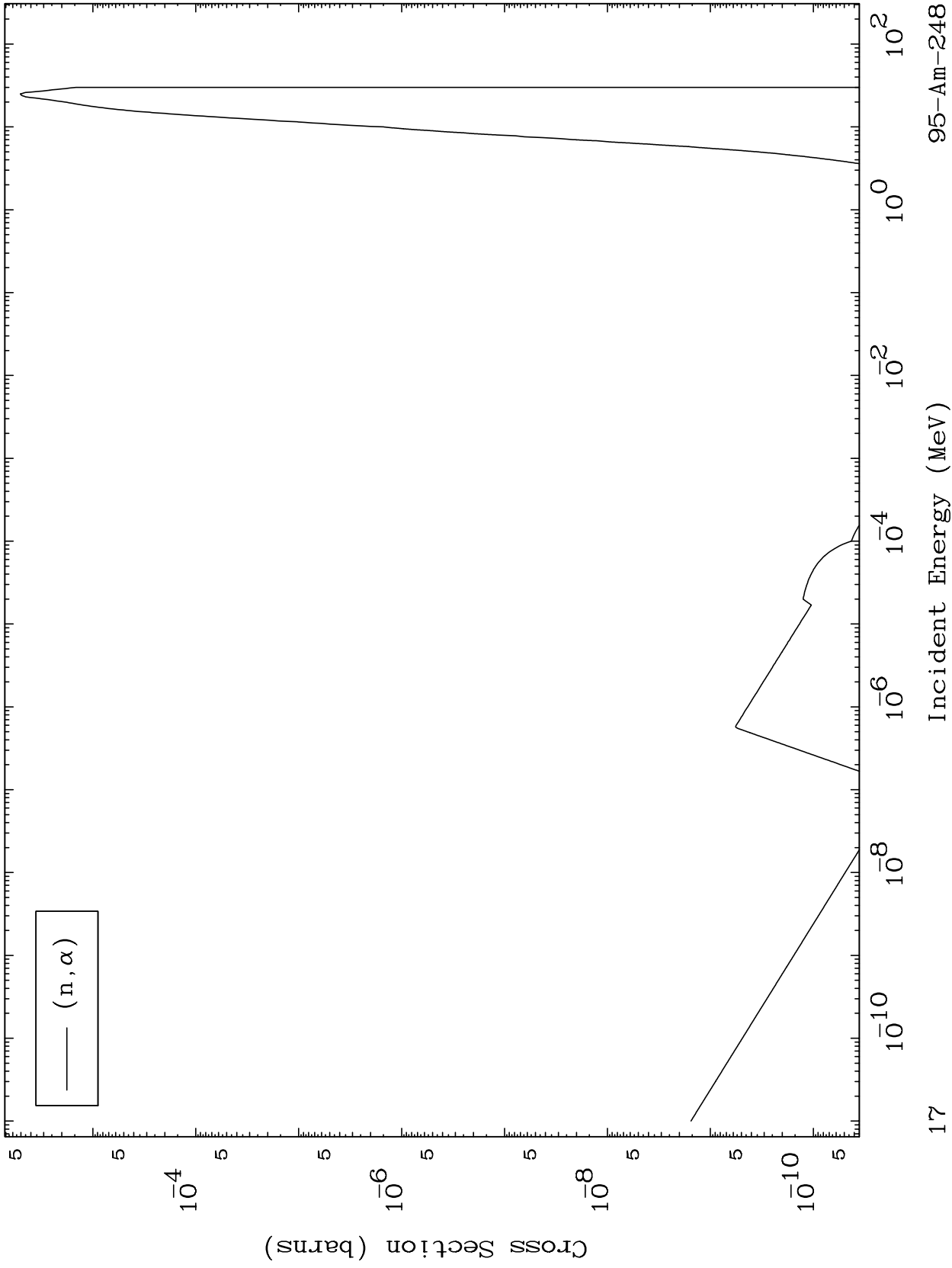
Incident Energy (MeV)

95-Am-248

MAT 9564

(n, α) Levels
293 Kelvin Cross Sections

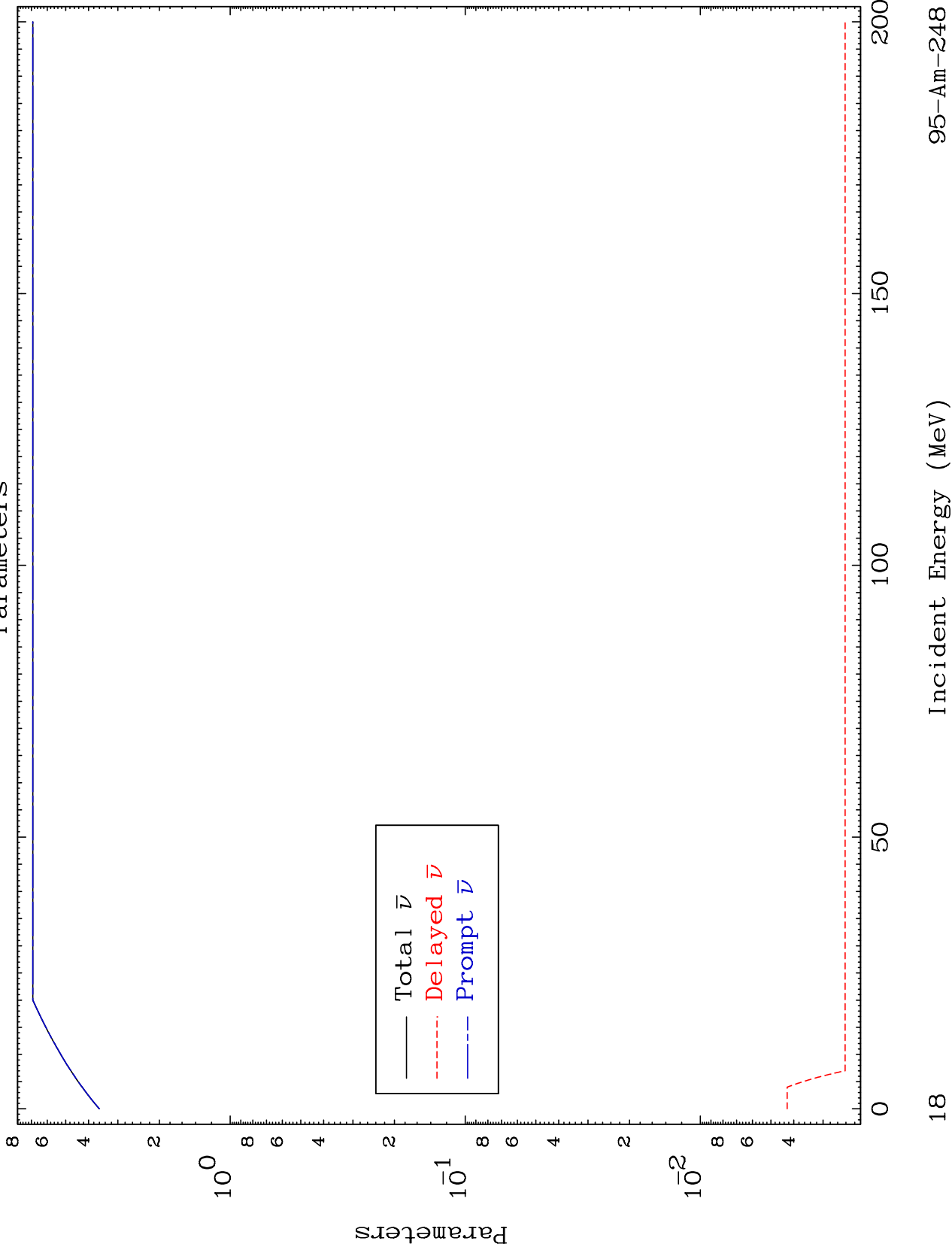
95-Am-248



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Energy Release
Parameters

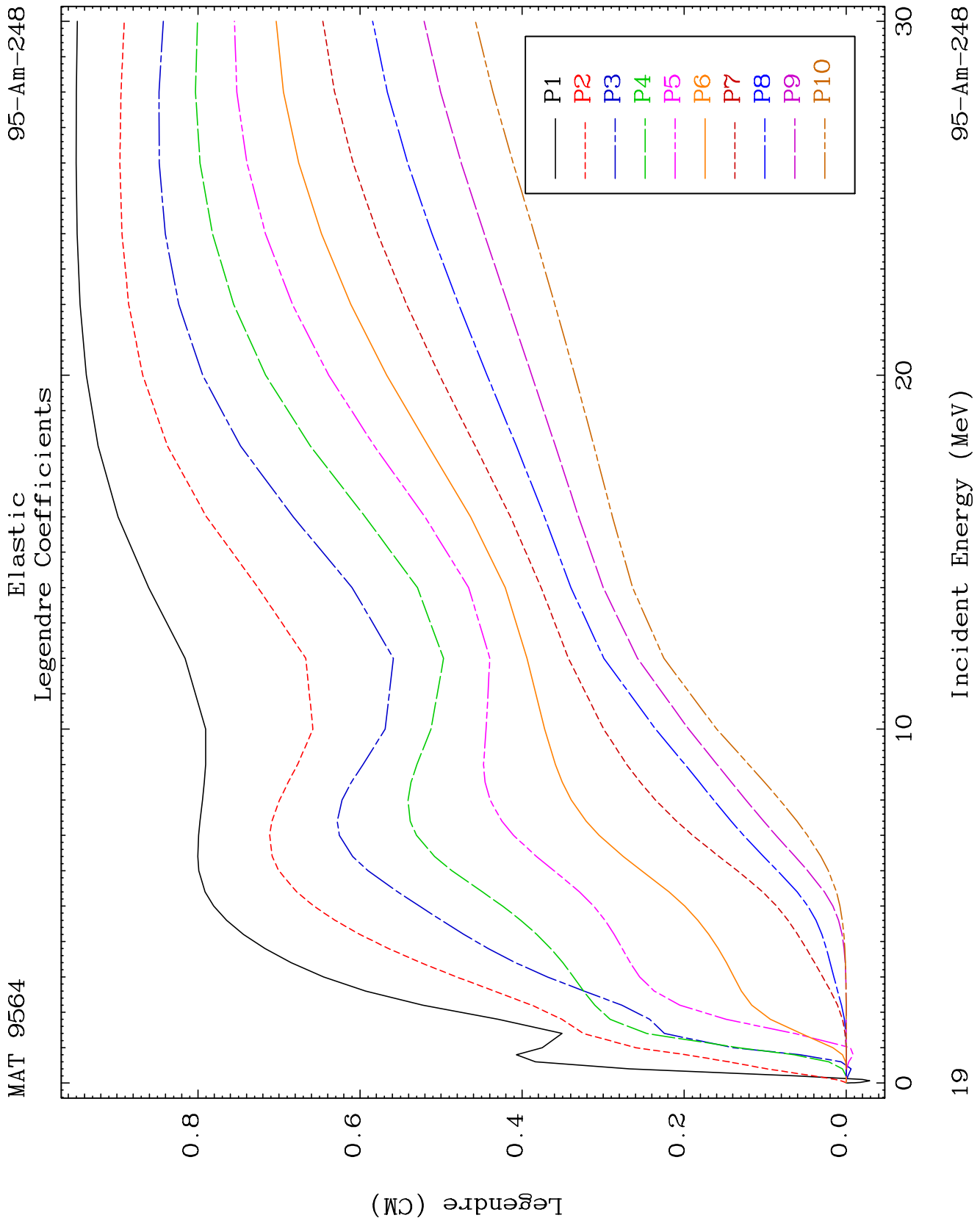
95-Am-248

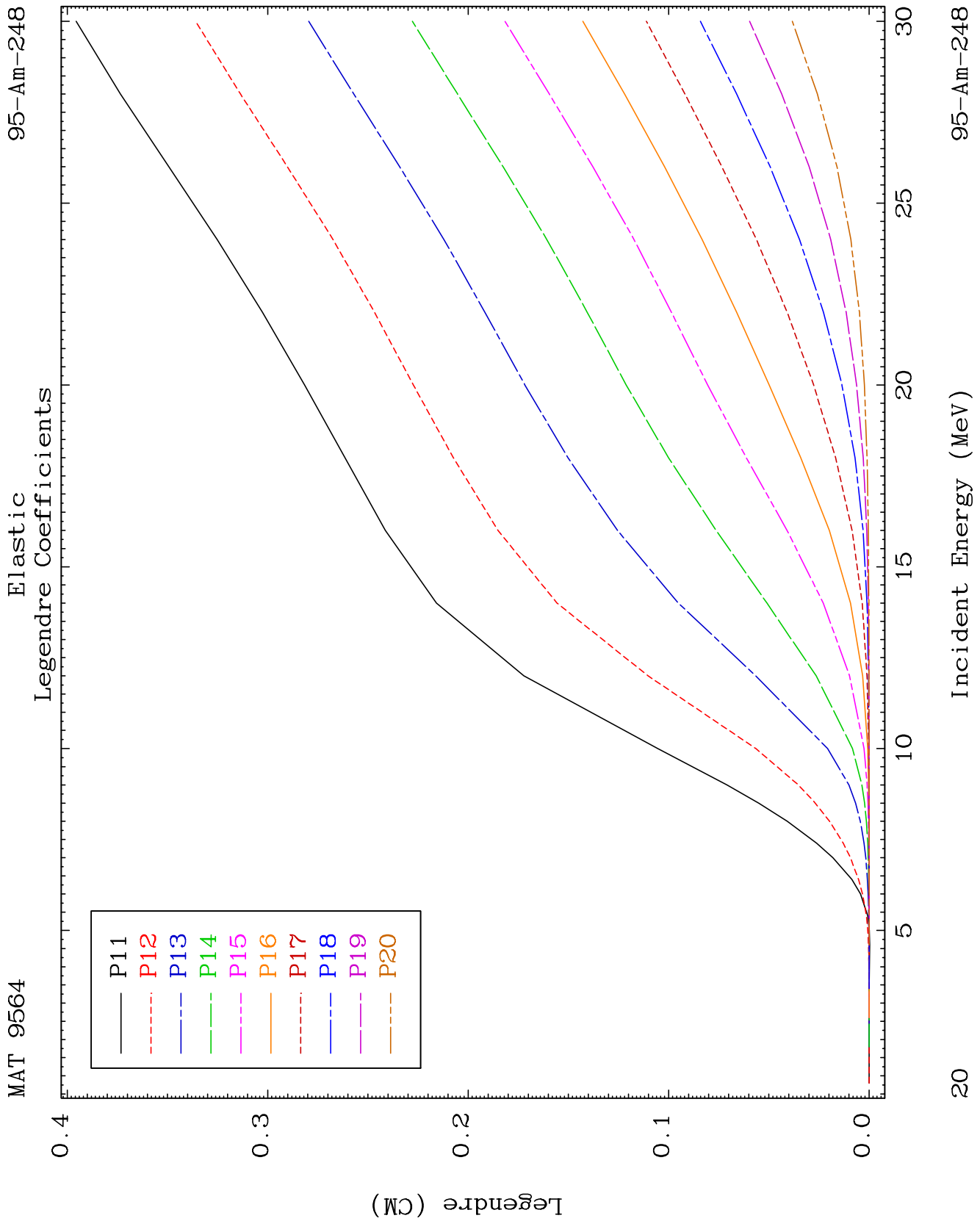


18

Incident Energy (MeV)

95-Am-248

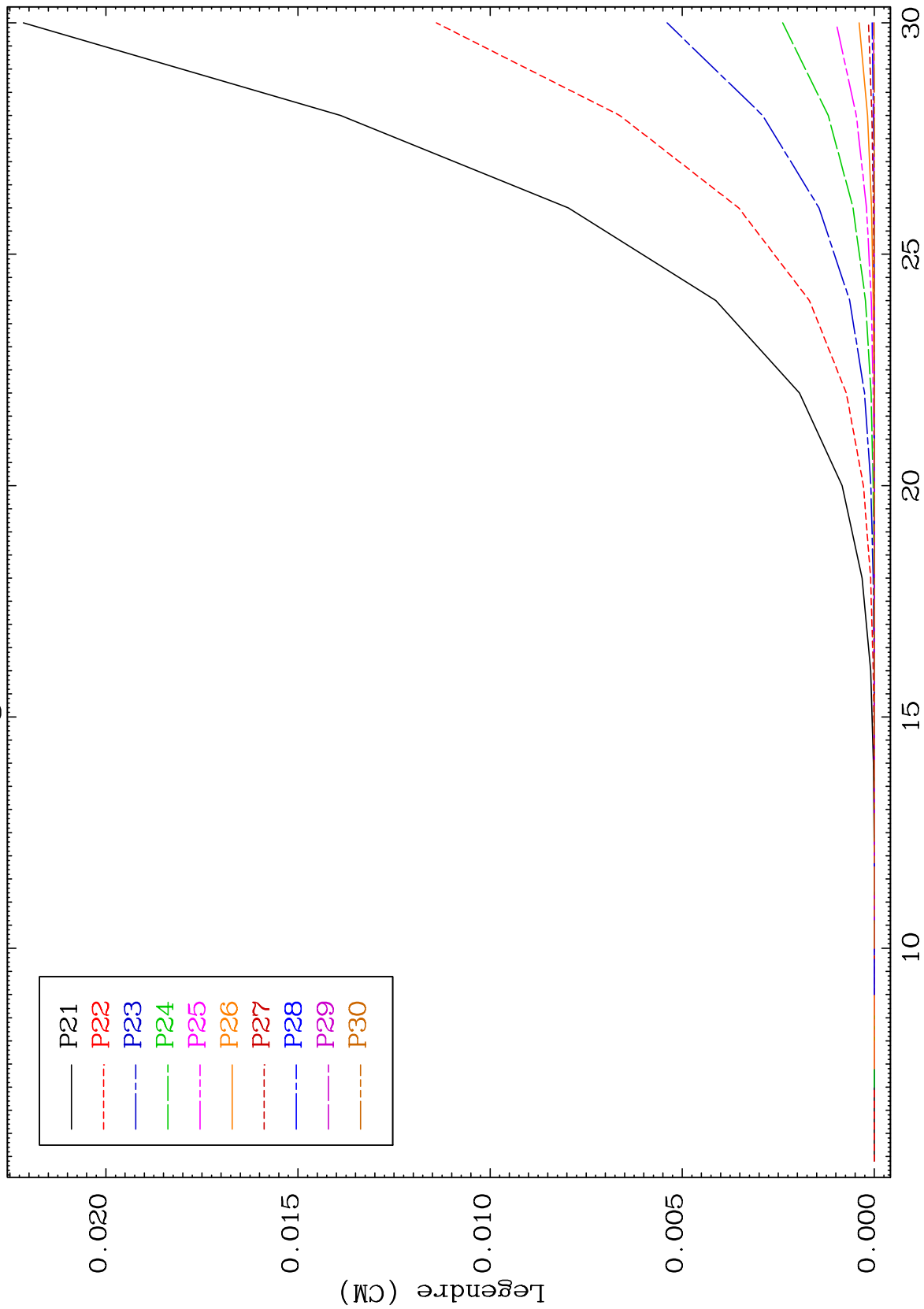




MAT 9564

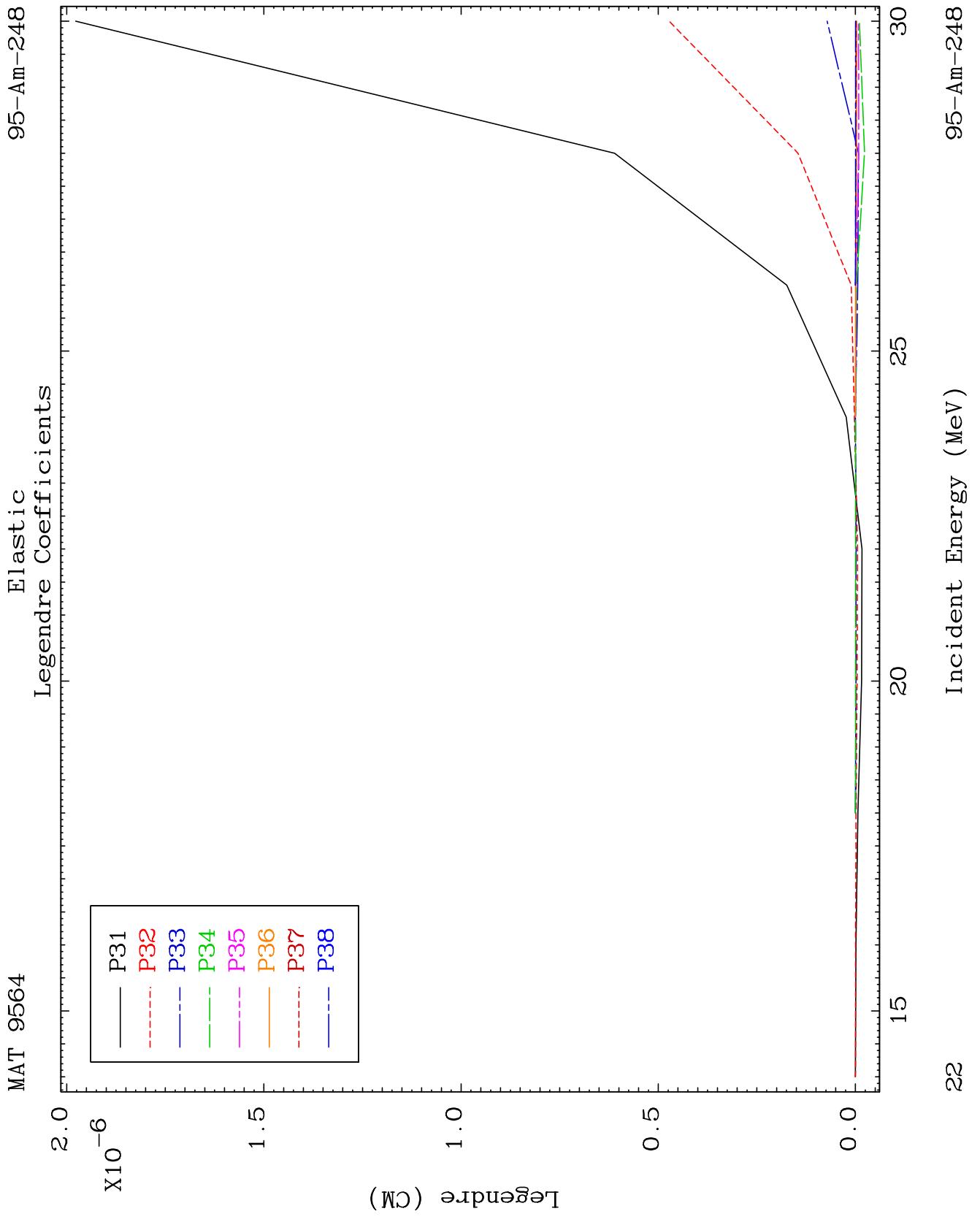
Elastic Legendre Coefficients

95-Am-248



21

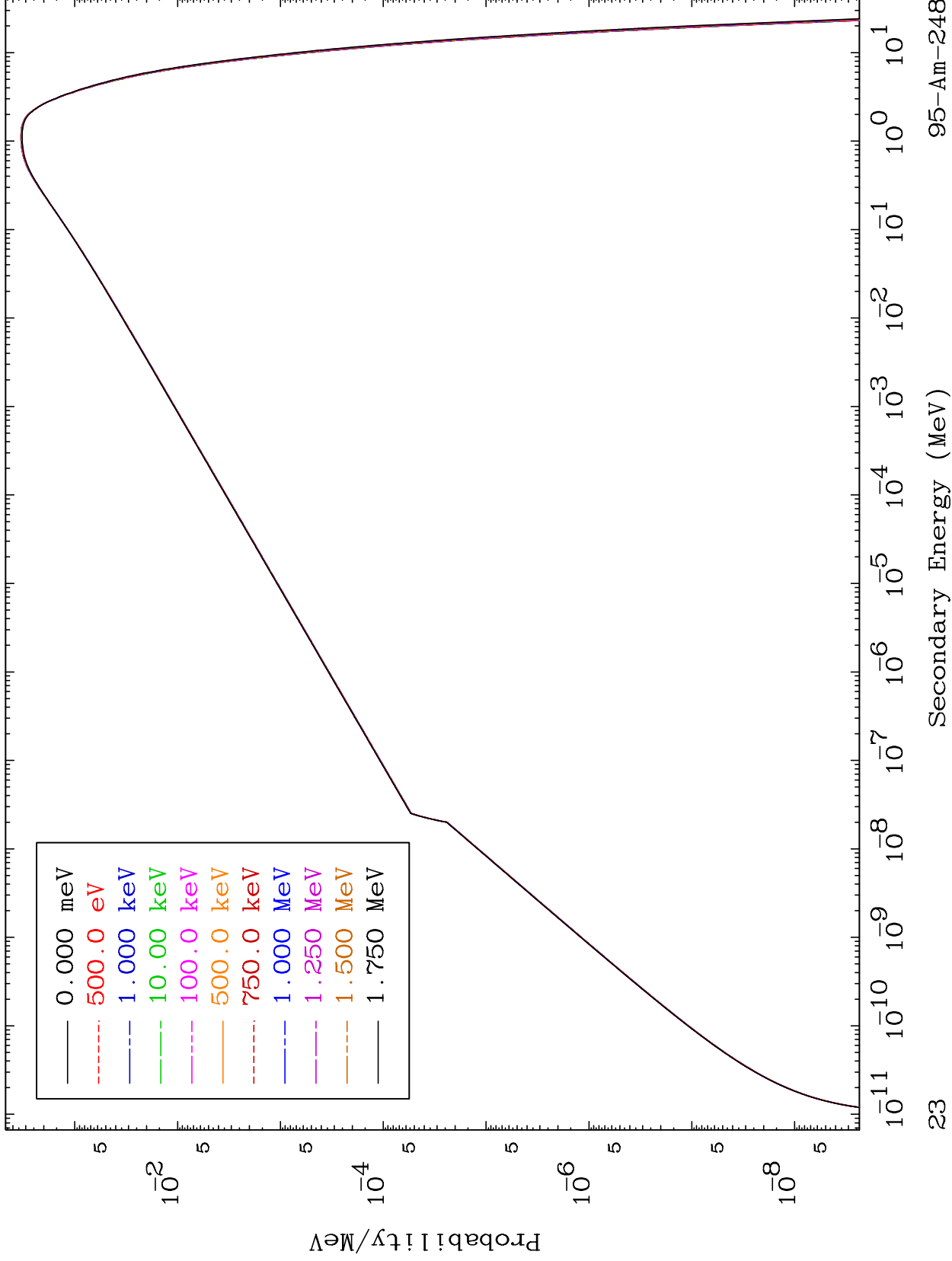
95-Am-248



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Fission Energy Distributions

95-Am-248



95-Am-248

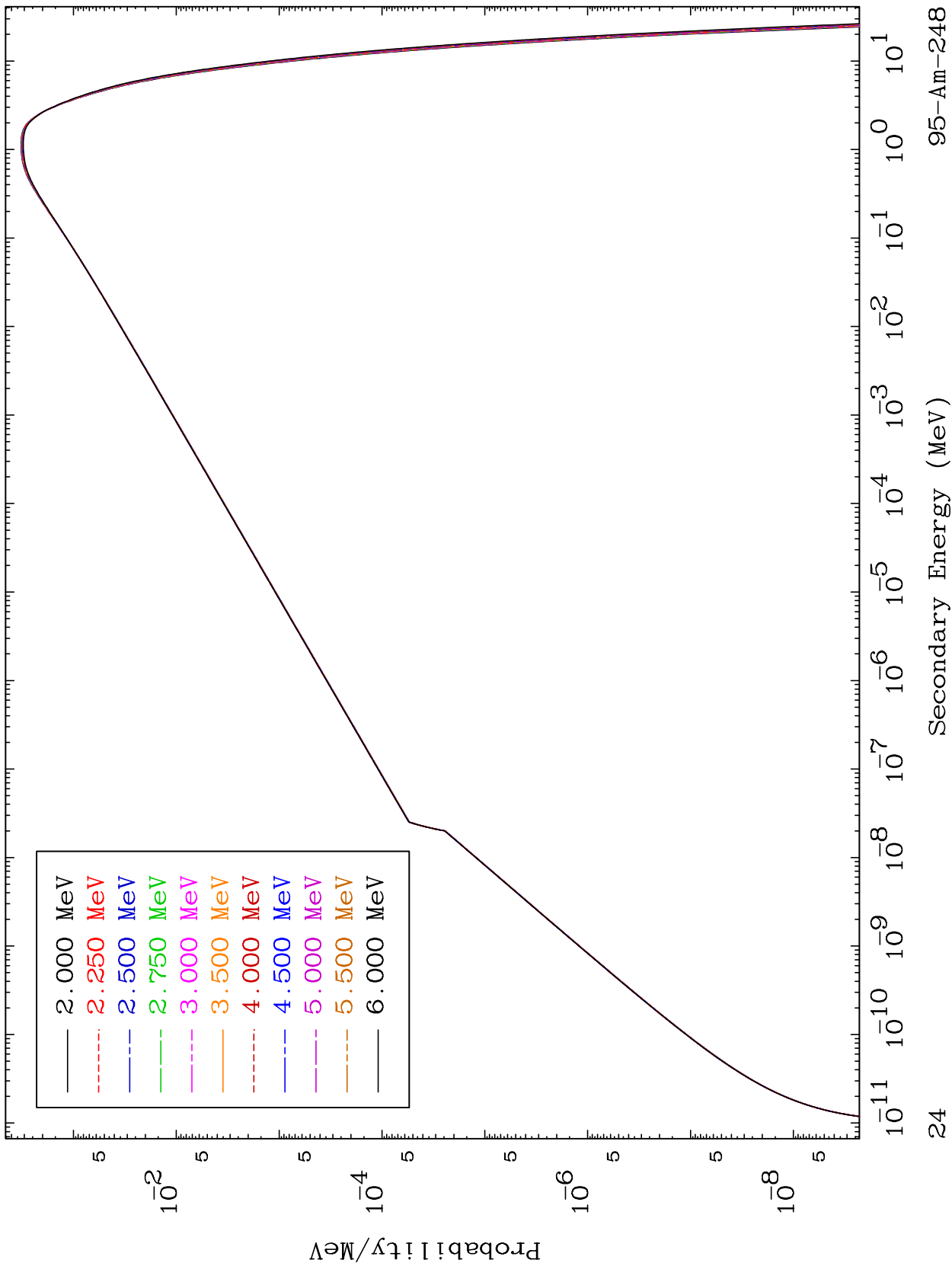
Secondary Energy (MeV)

23

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Fission Energy Distributions

95-Am-248



95-Am-248

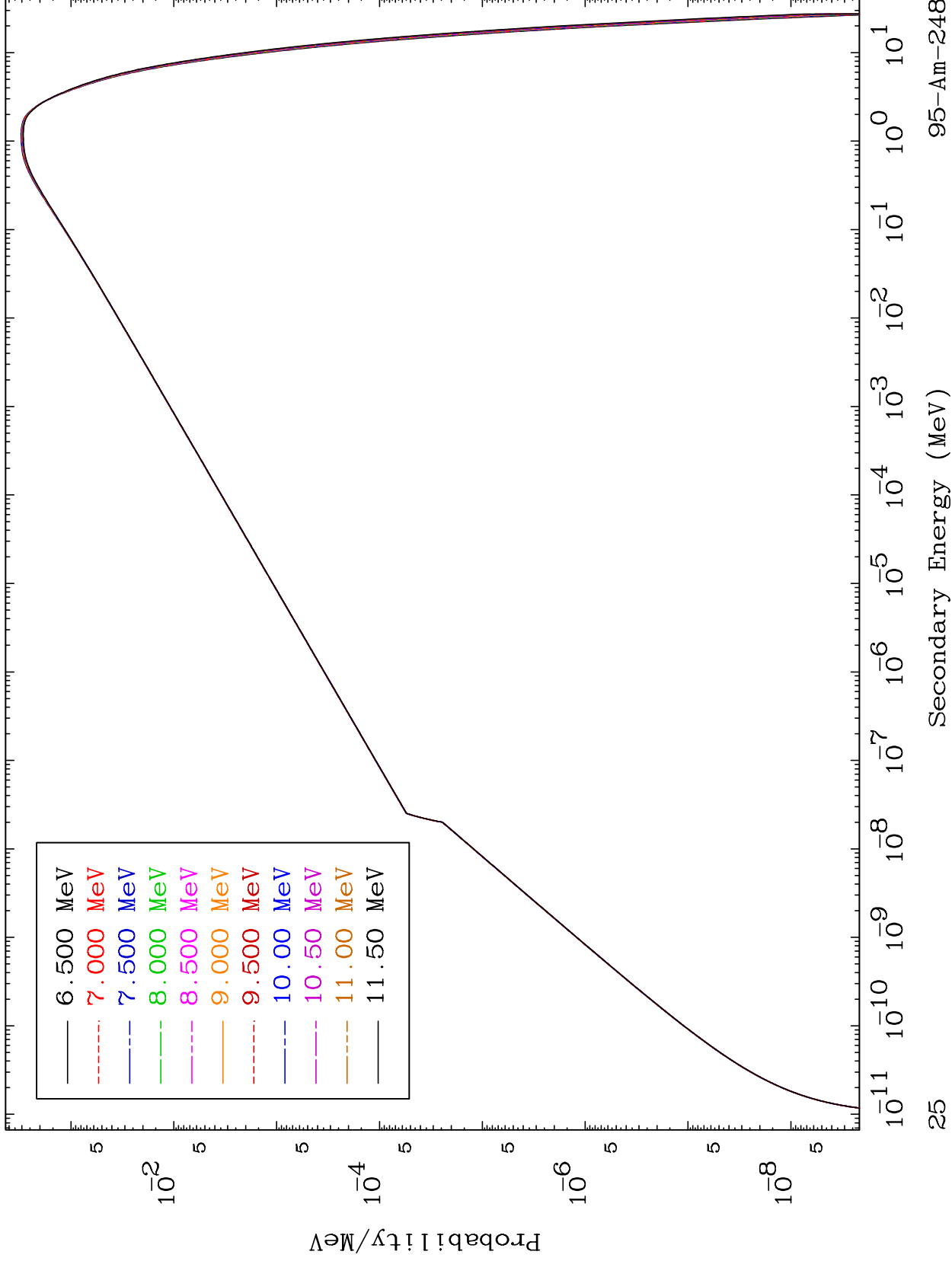
Secondary Energy (MeV)

24

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Fission Energy Distributions

95-Am-248



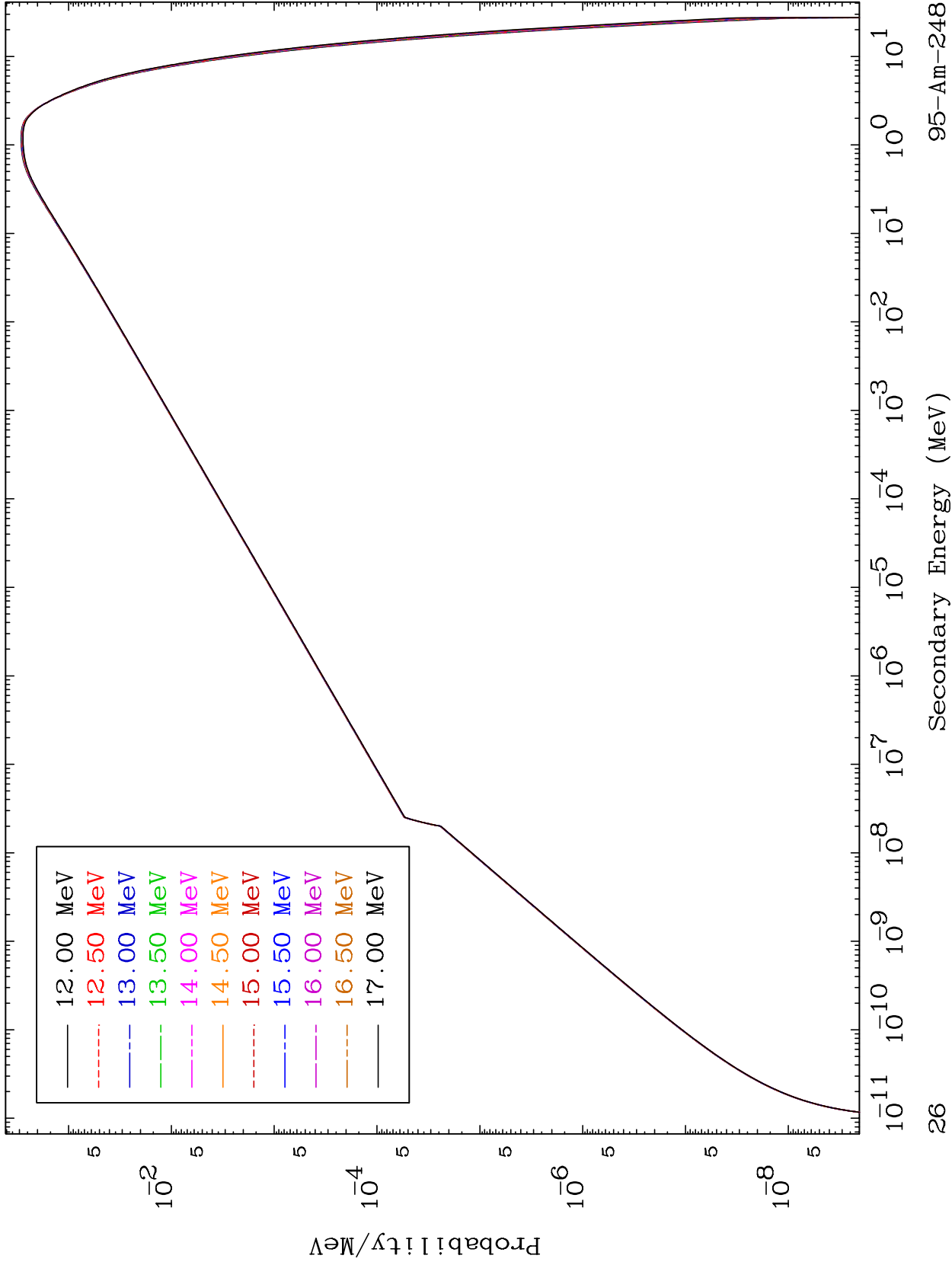
95-Am-248

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Fission Energy Distributions

95-Am-248



95-Am-248

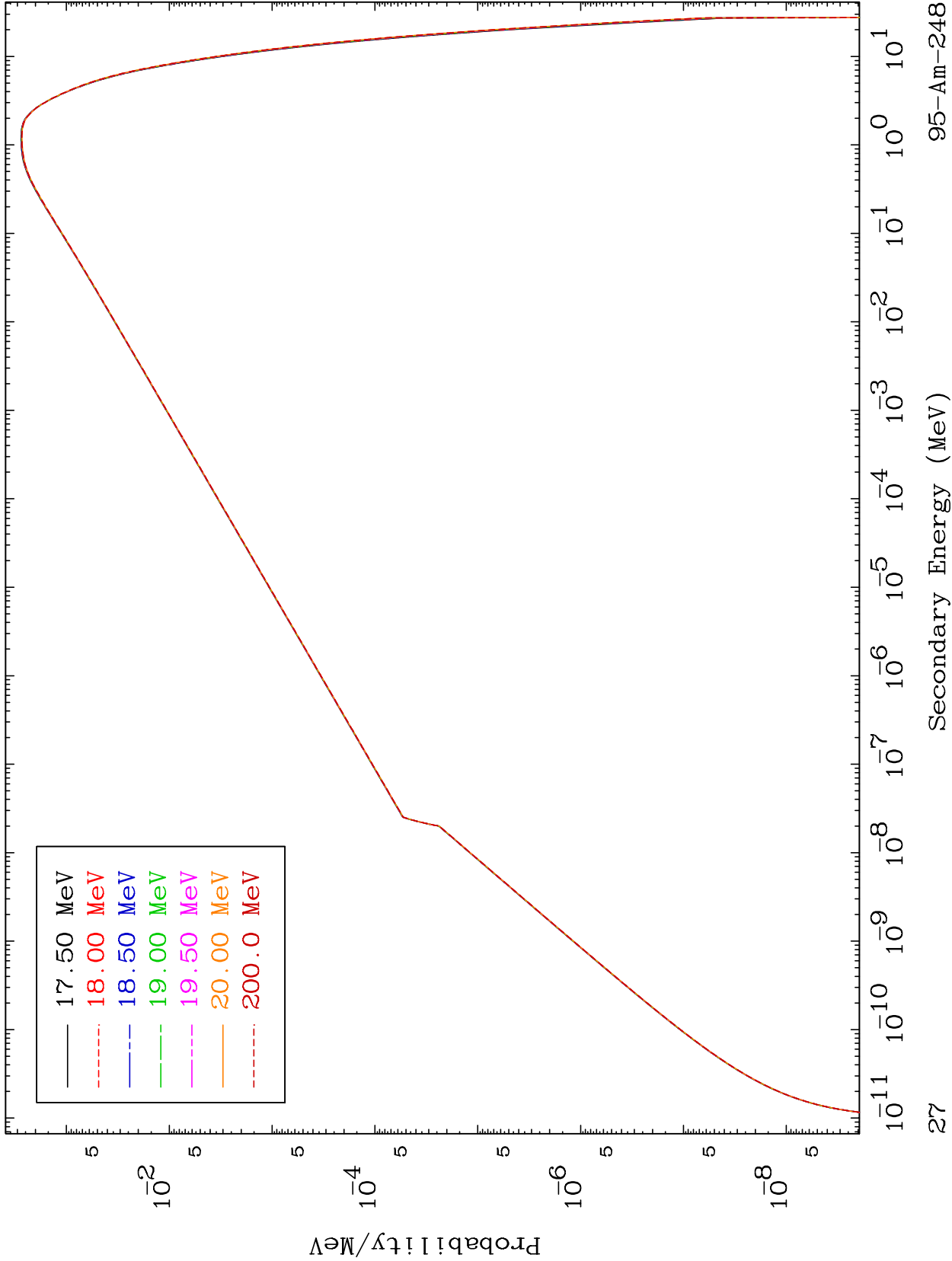
Secondary Energy (MeV)

26

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Fission Energy Distributions

95-Am-248

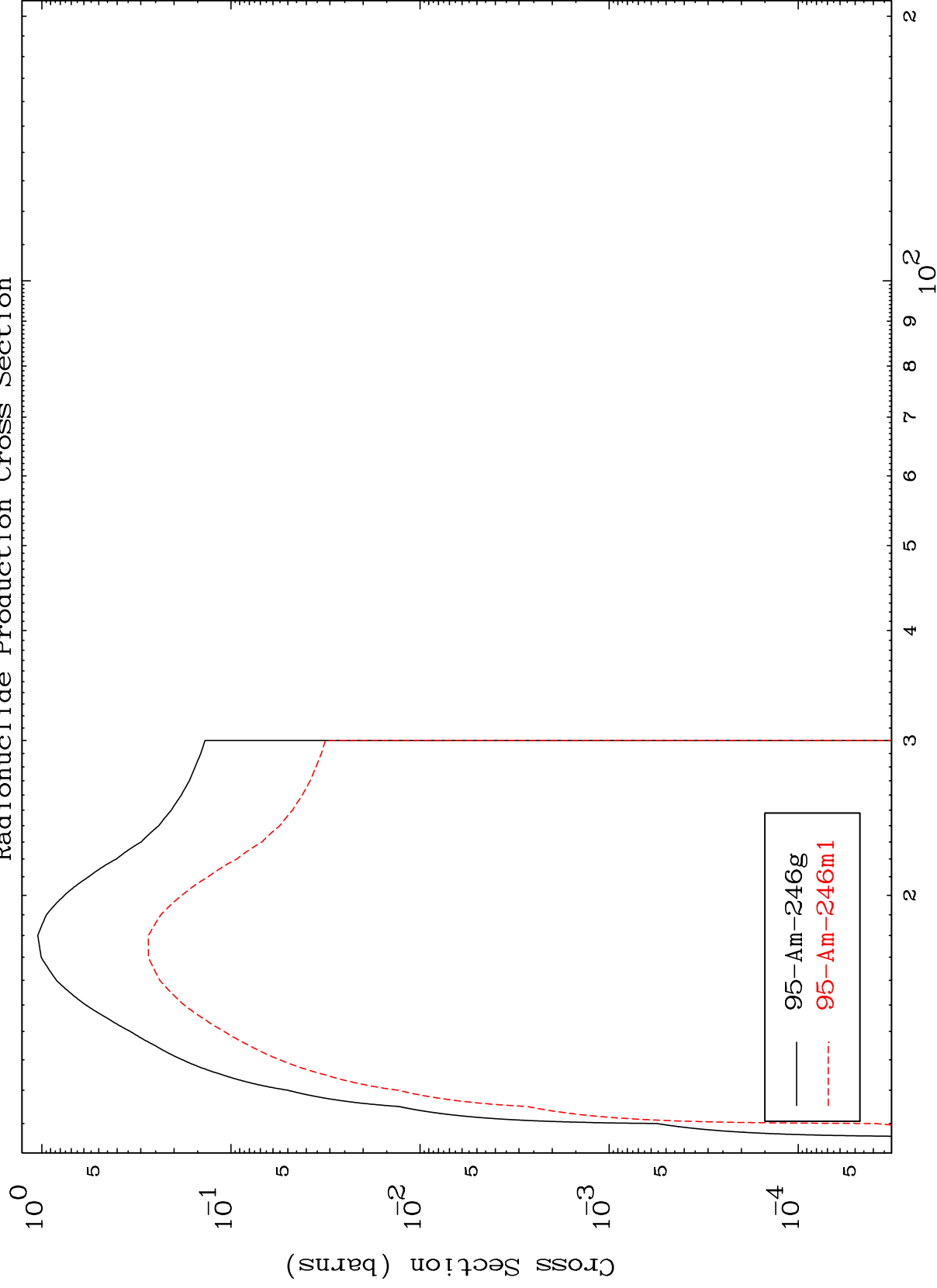


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(n,3n)

95-Am-248

Radionuclide Production Cross Section



28

Incident Energy (MeV)

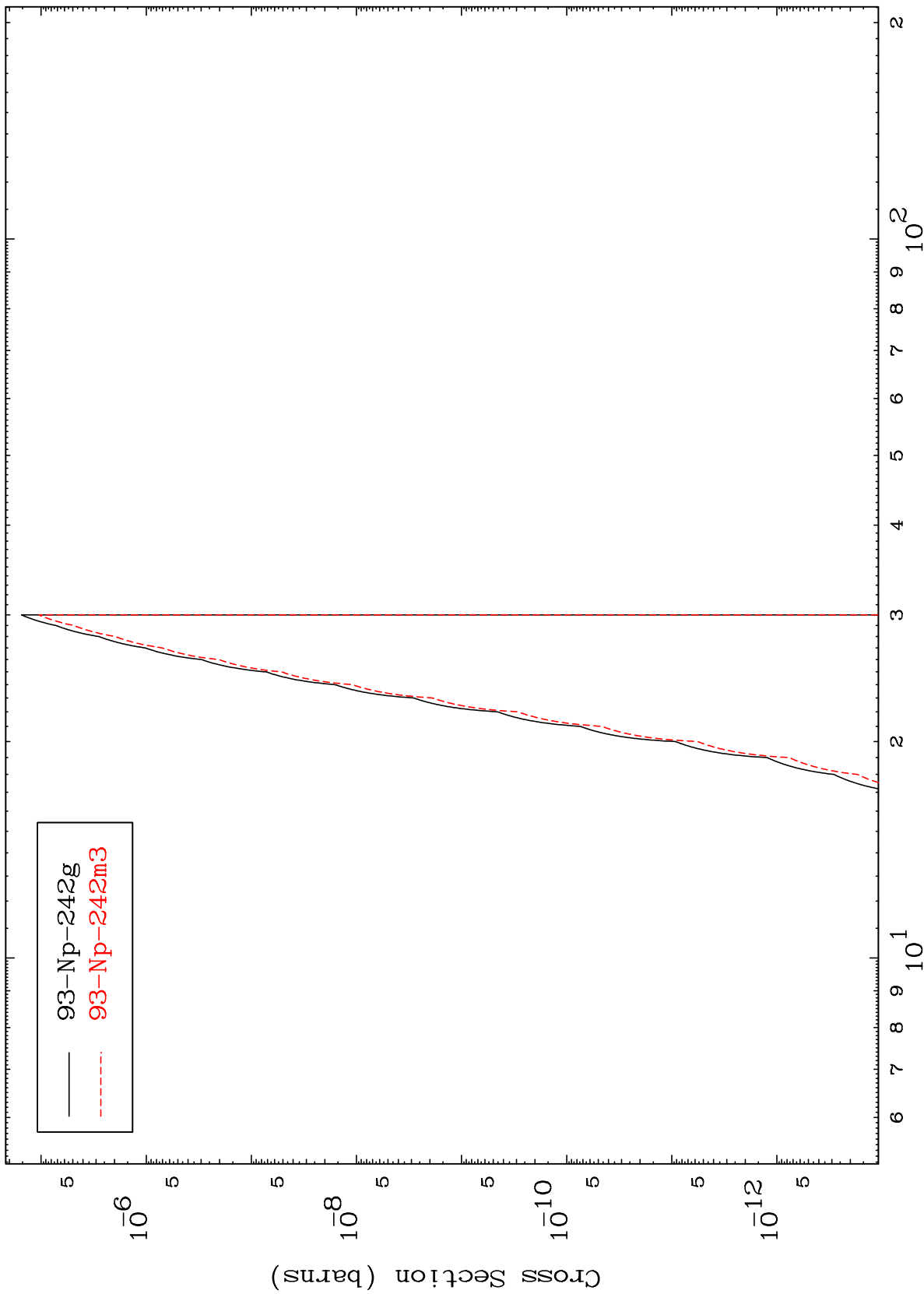
95-Am-248

MAT 9564

(n,3n) α

95-Am-248

Radionuclide Production Cross Section



29

Incident Energy (MeV)

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