

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

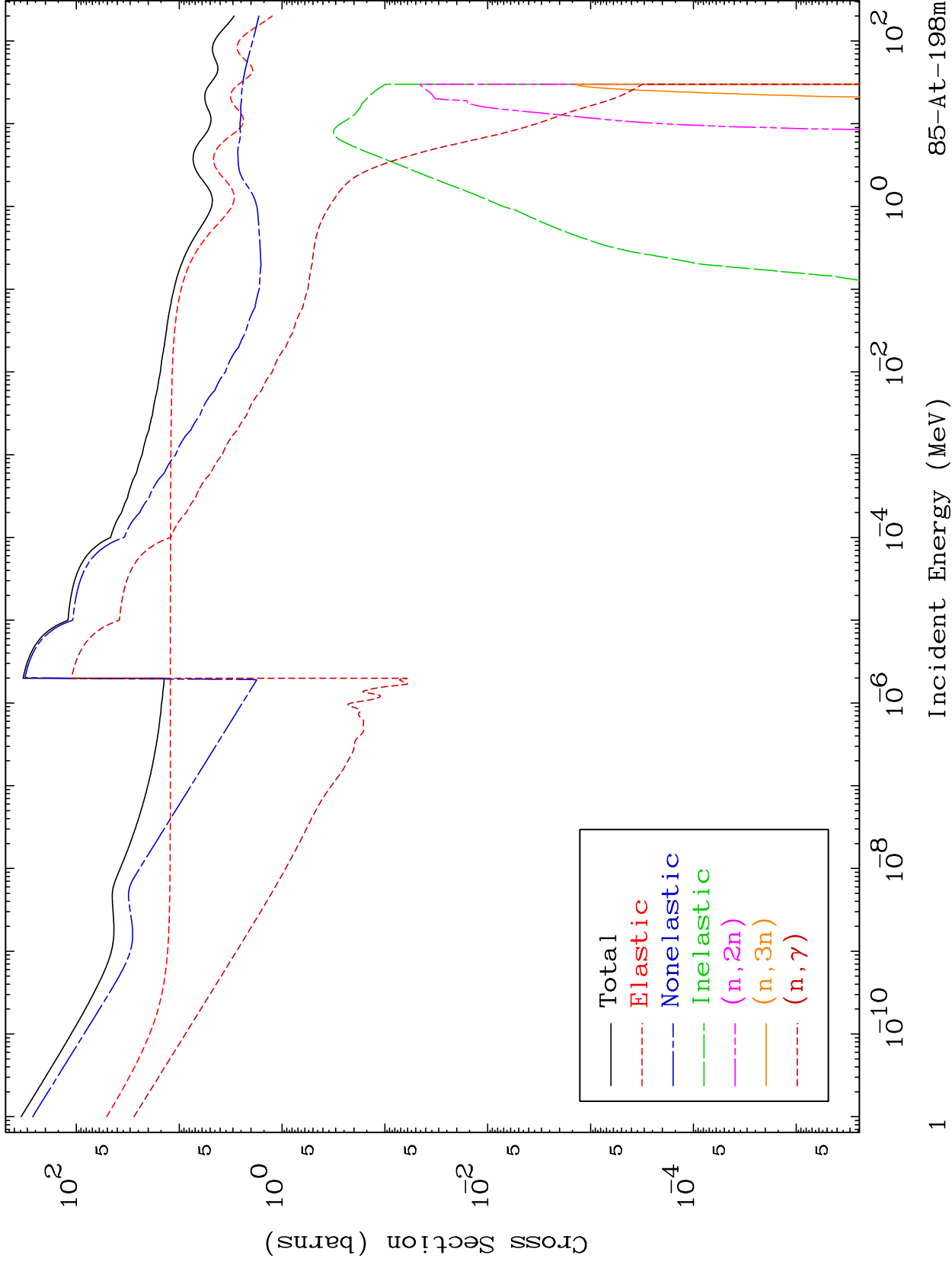
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8511

Neutron Major  
293 Kelvin Cross Sections

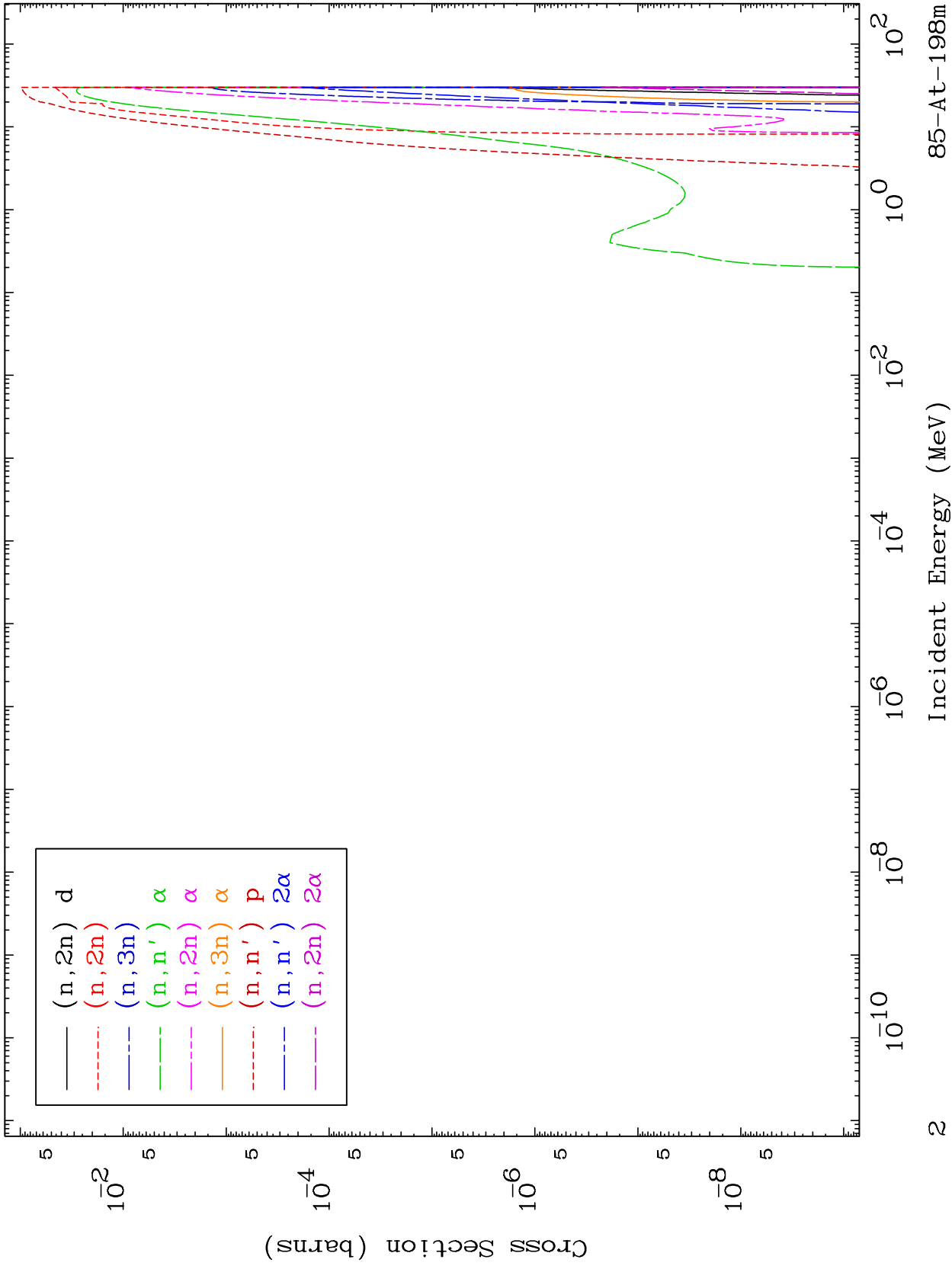
85-At-198m



MAT 8511

Neutron Absorption  
293 Kelvin Cross Sections

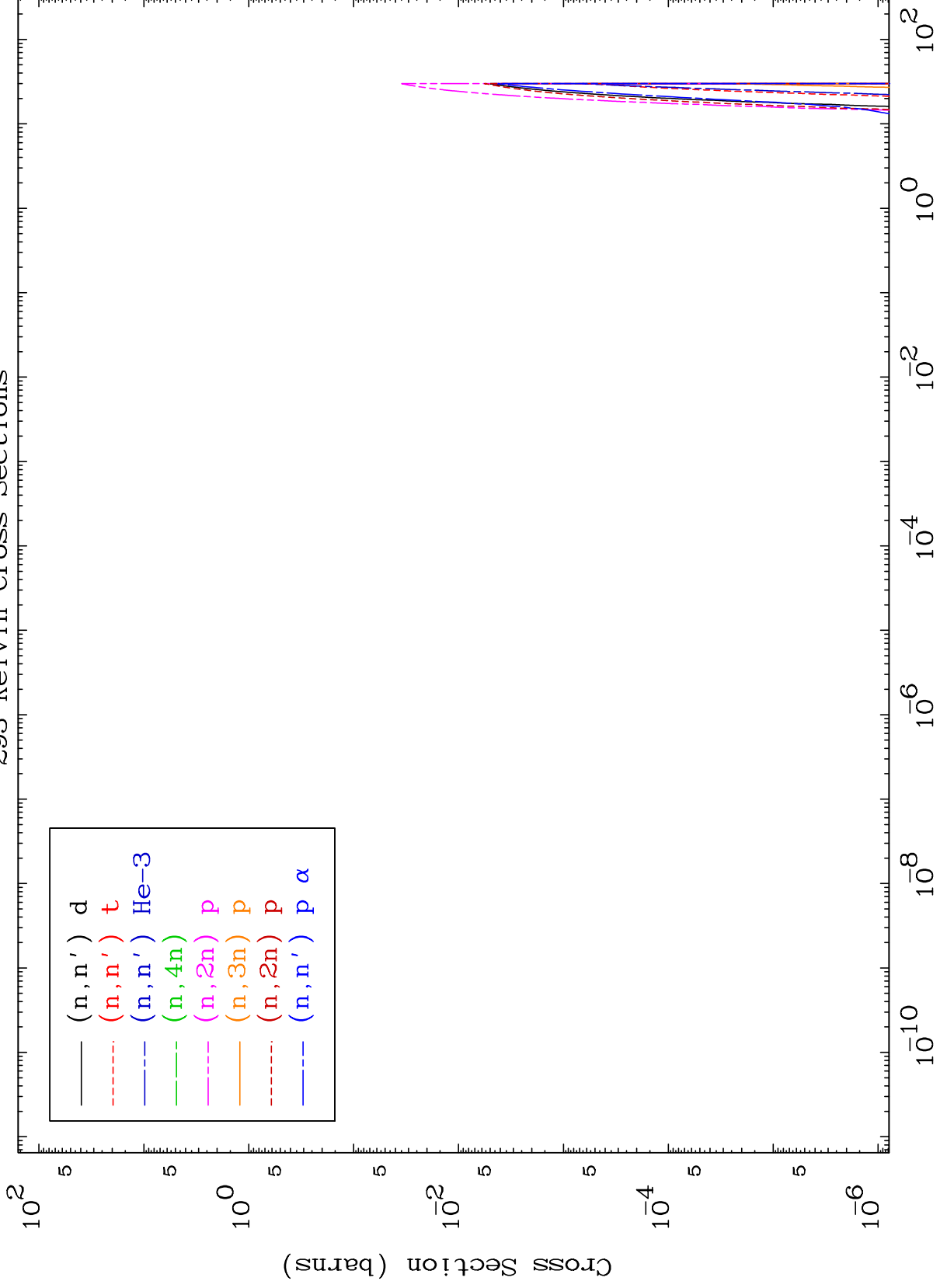
85-At-198m



MAT 8511

Neutron Absorption  
293 Kelvin Cross Sections

85-At-198m



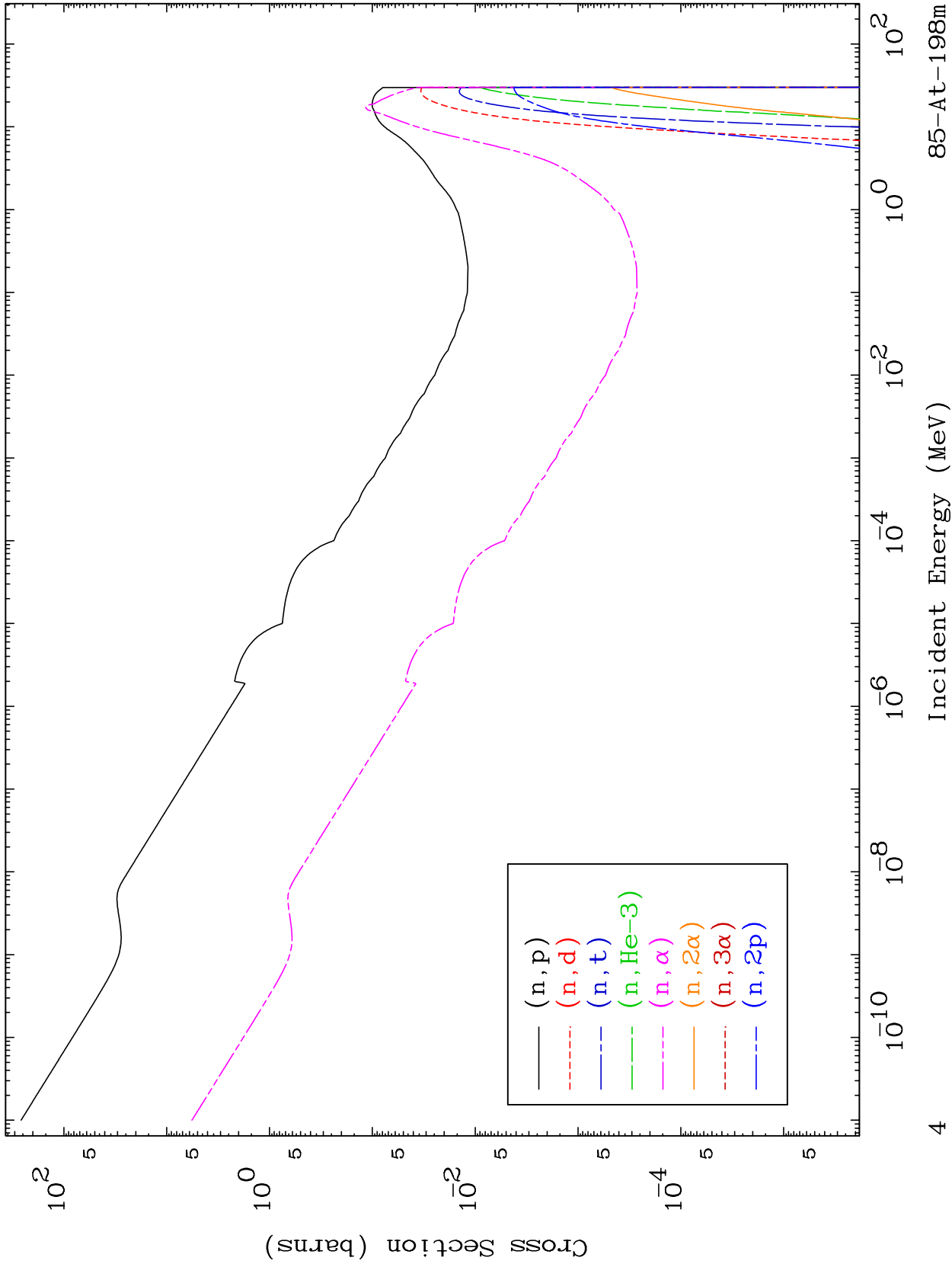
85-At-198m

Incident Energy (MeV)

MAT 8511

Neutron Absorption  
293 Kelvin Cross Sections

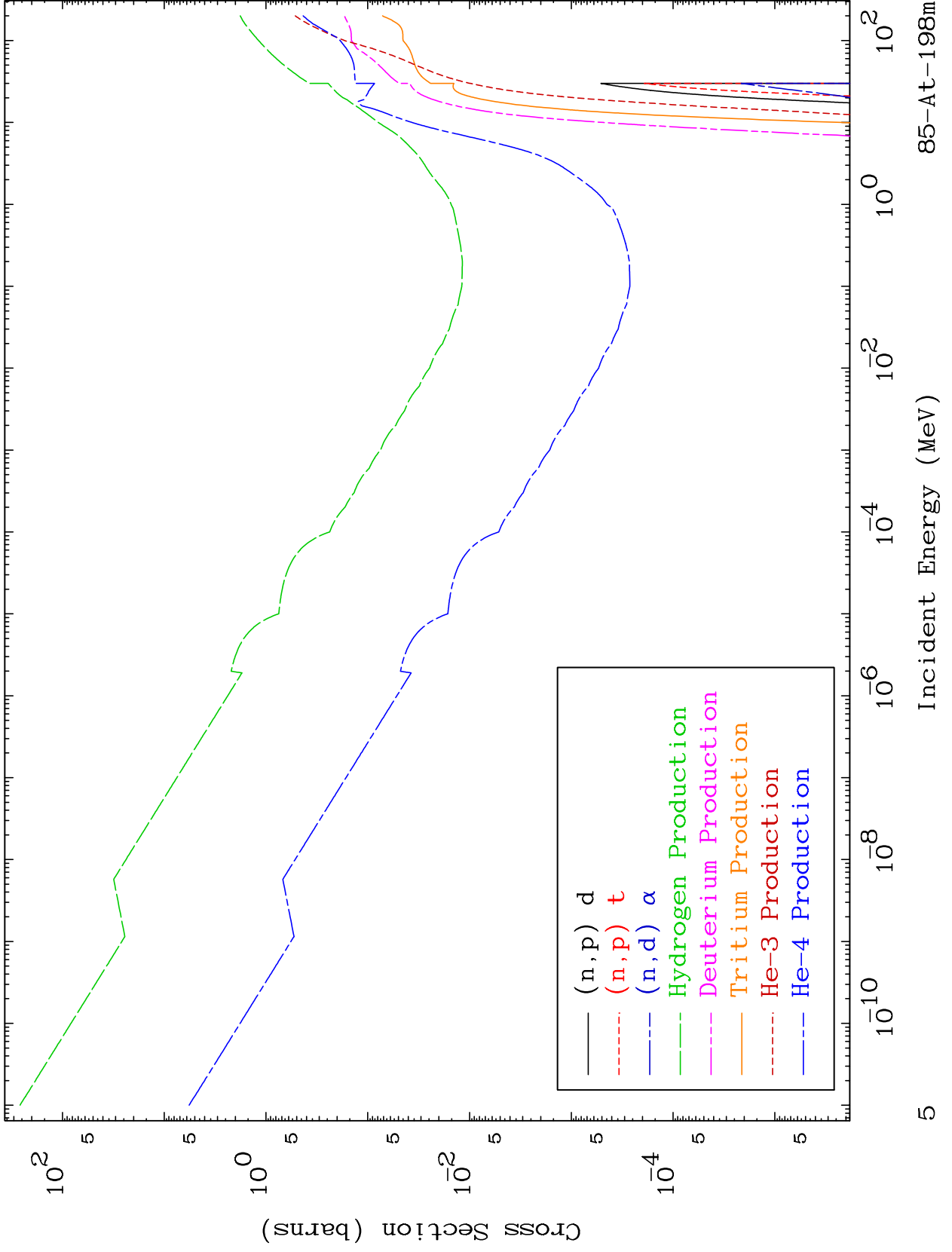
85-At-198m



MAT 8511

Neutron Absorption  
293 Kelvin Cross Sections

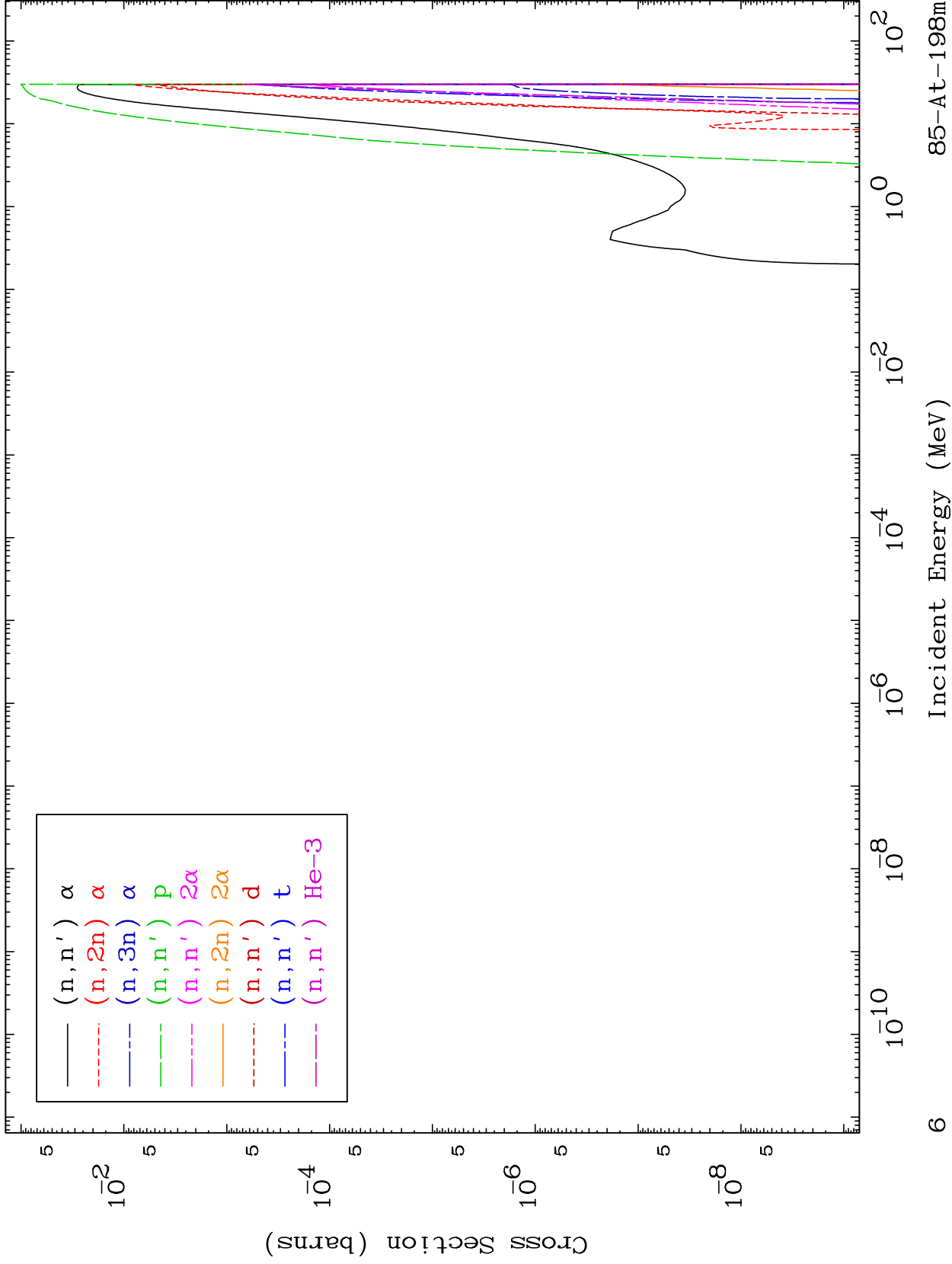
85-At-198m



MAT 8511

Charged Particle  
293 Kelvin Cross Sections

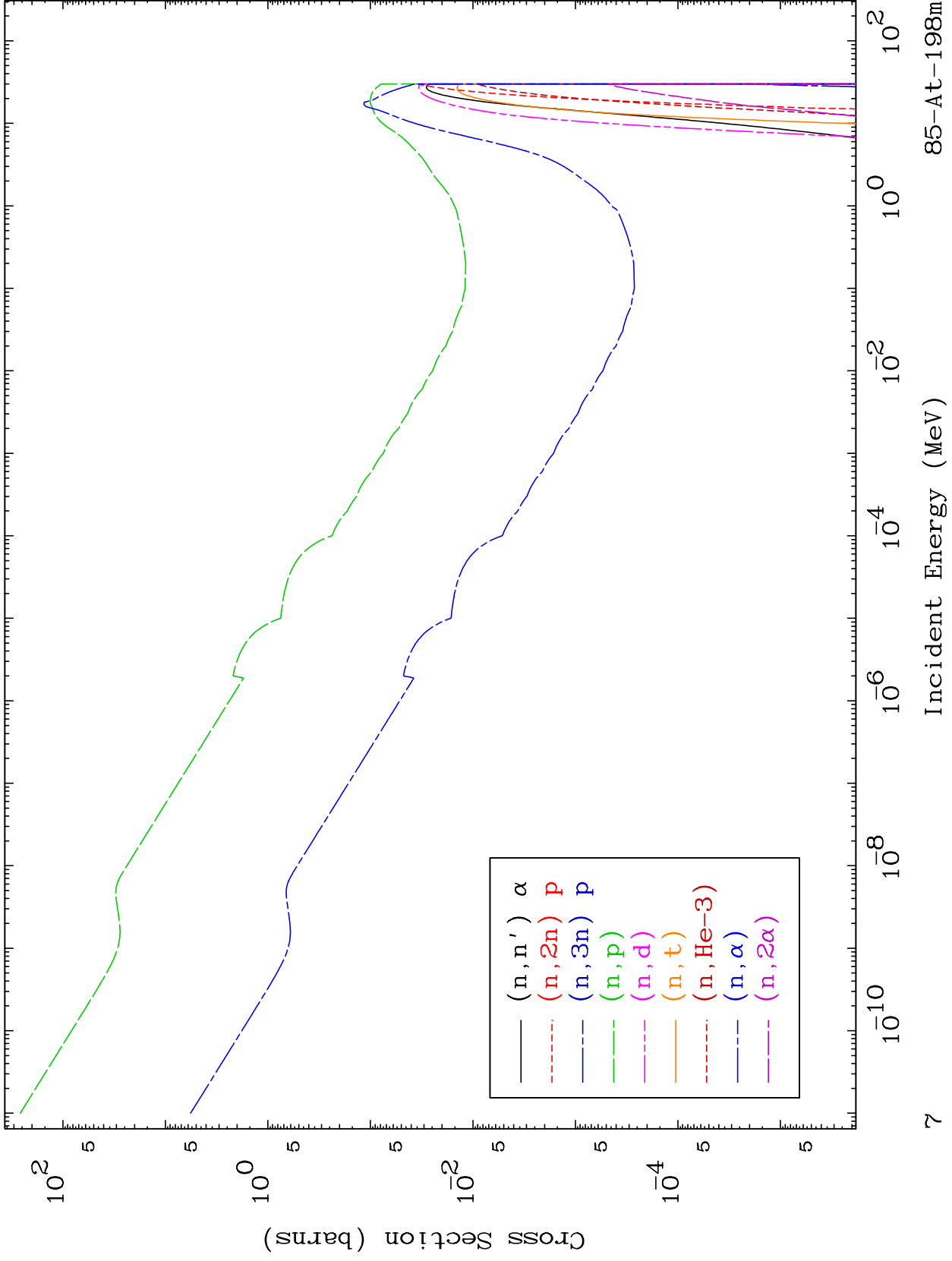
85-At-198m



MAT 8511

Charged Particle  
293 Kelvin Cross Sections

85-At-198m

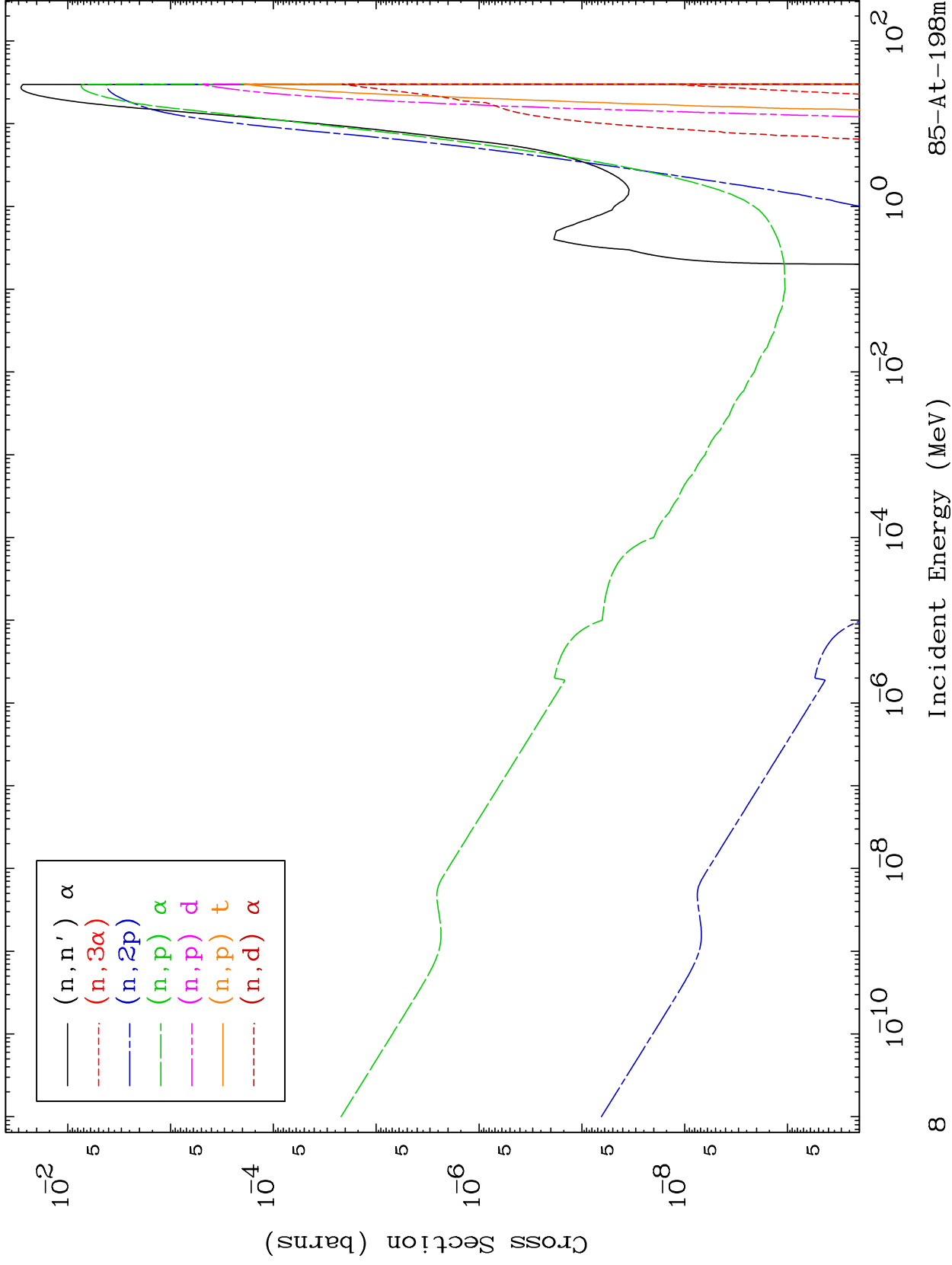


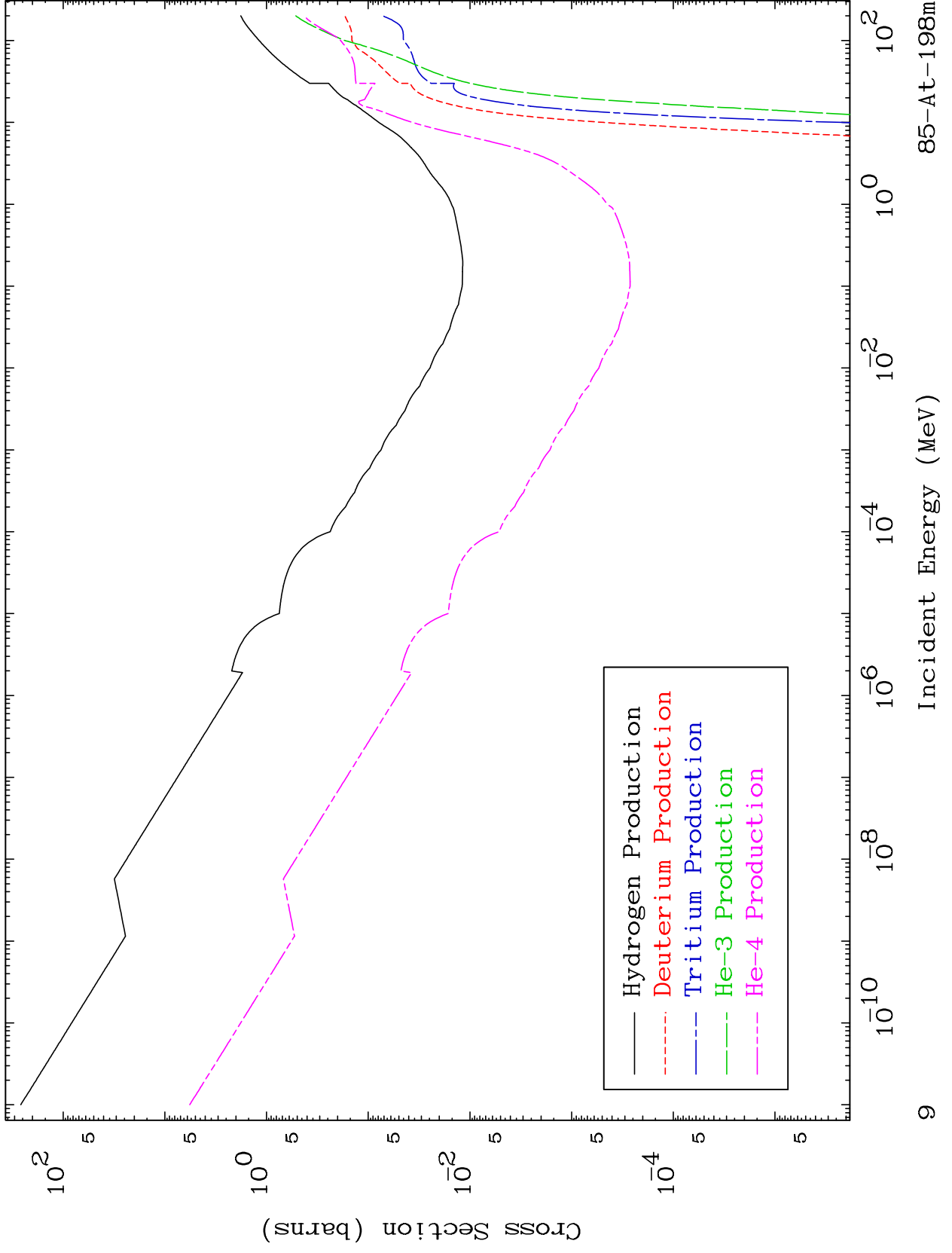


MAT 8511

Charged Particle  
293 Kelvin Cross Sections

85-At-198m

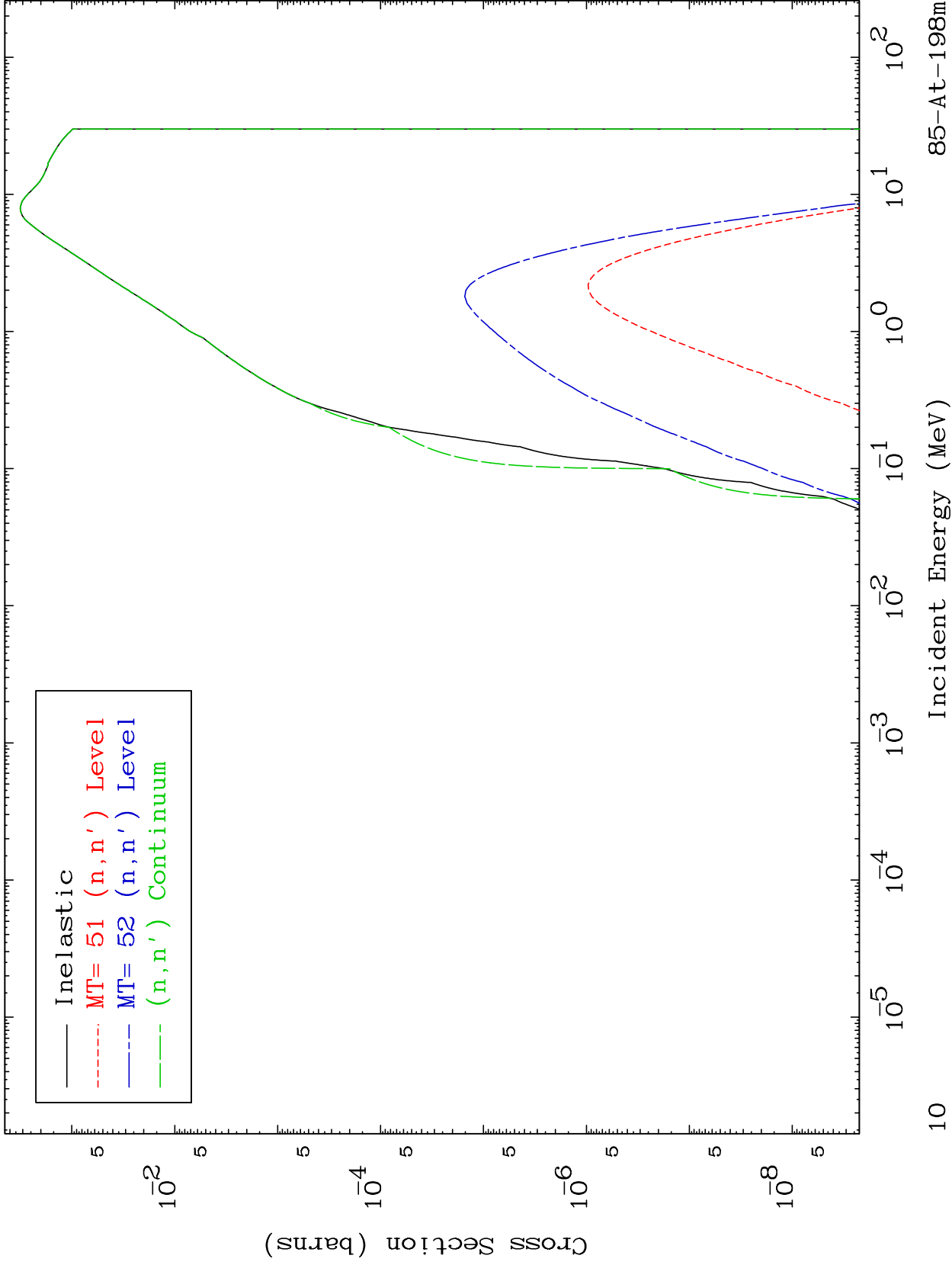




MAT 8511

(n,n') Levels  
293 Kelvin Cross Sections

85-At-198m



10

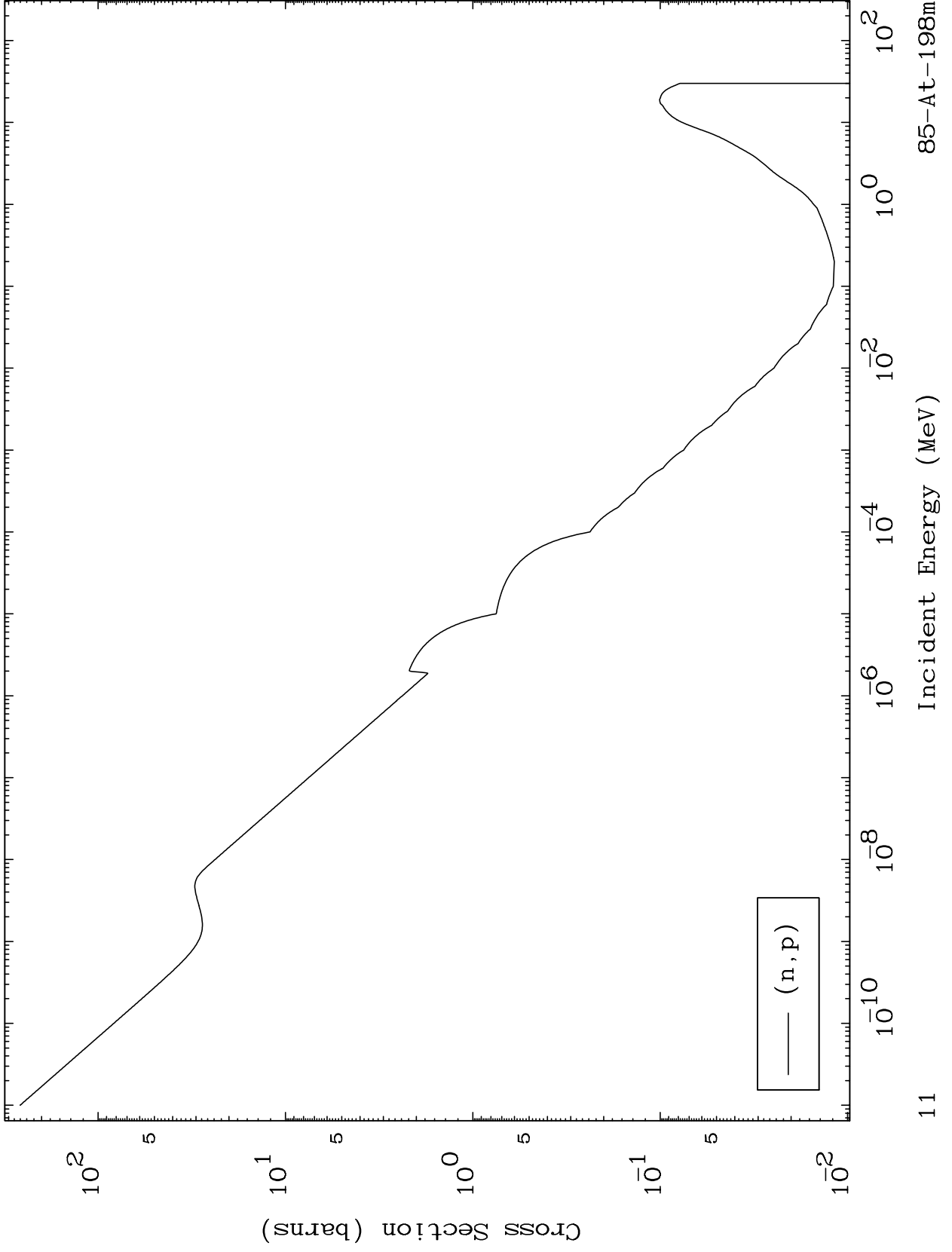
Incident Energy (MeV)

85-At-198m

MAT 8511

(n,p) Levels  
293 Kelvin Cross Sections

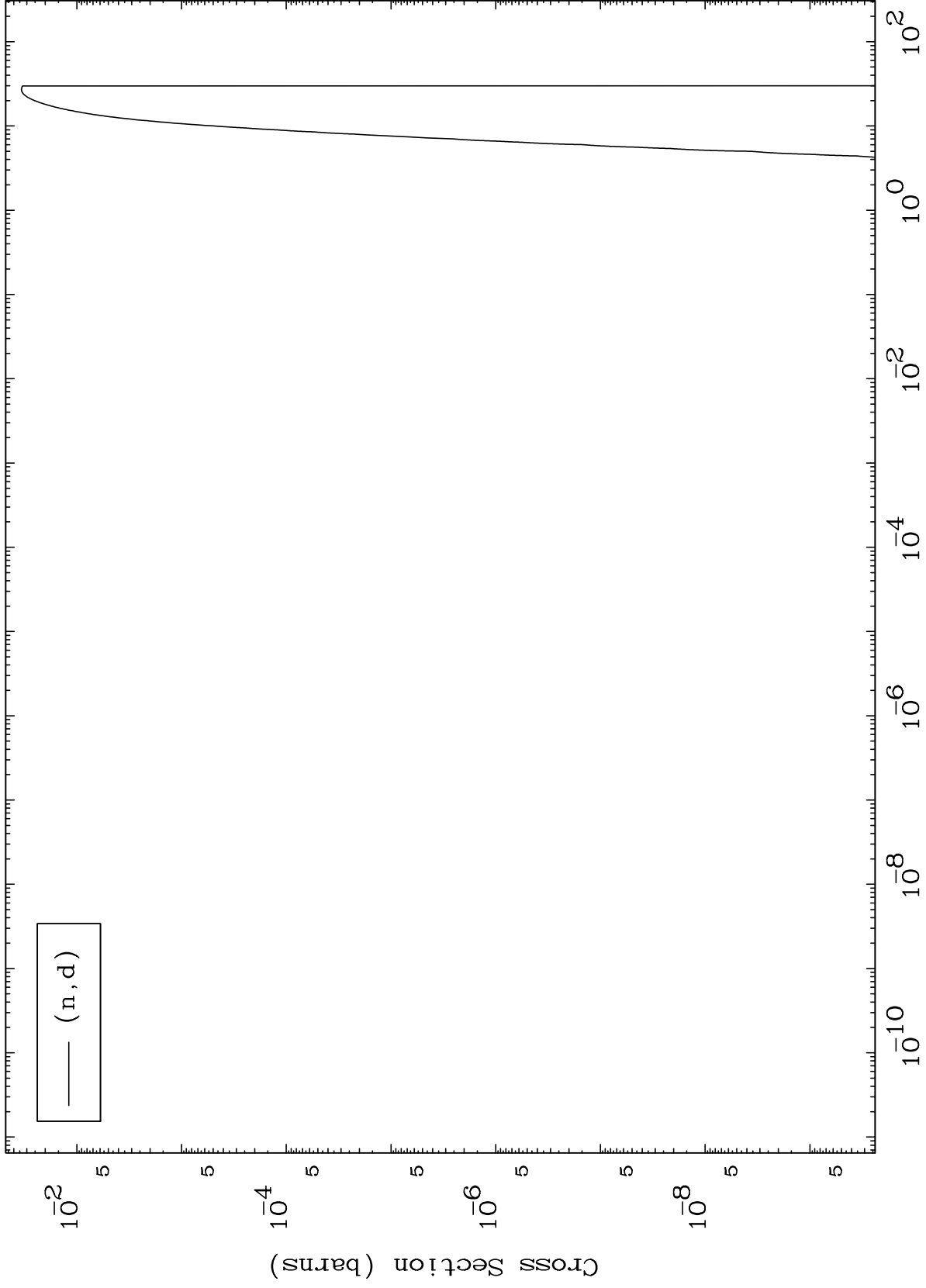
85-At-198m



MAT 8511

(n,d) Levels  
293 Kelvin Cross Sections

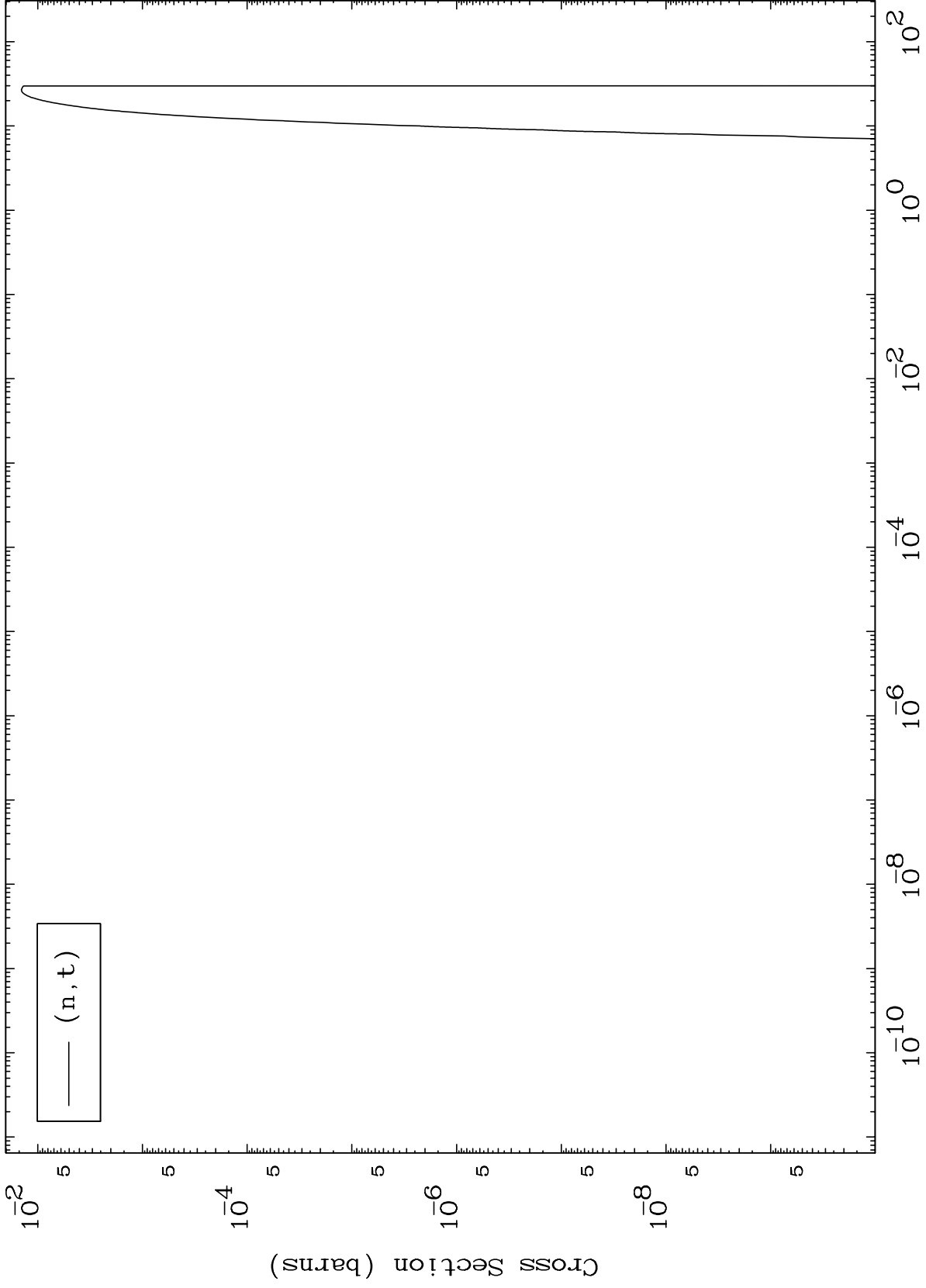
85-At-198m



MAT 8511

(n,t) Levels  
293 Kelvin Cross Sections

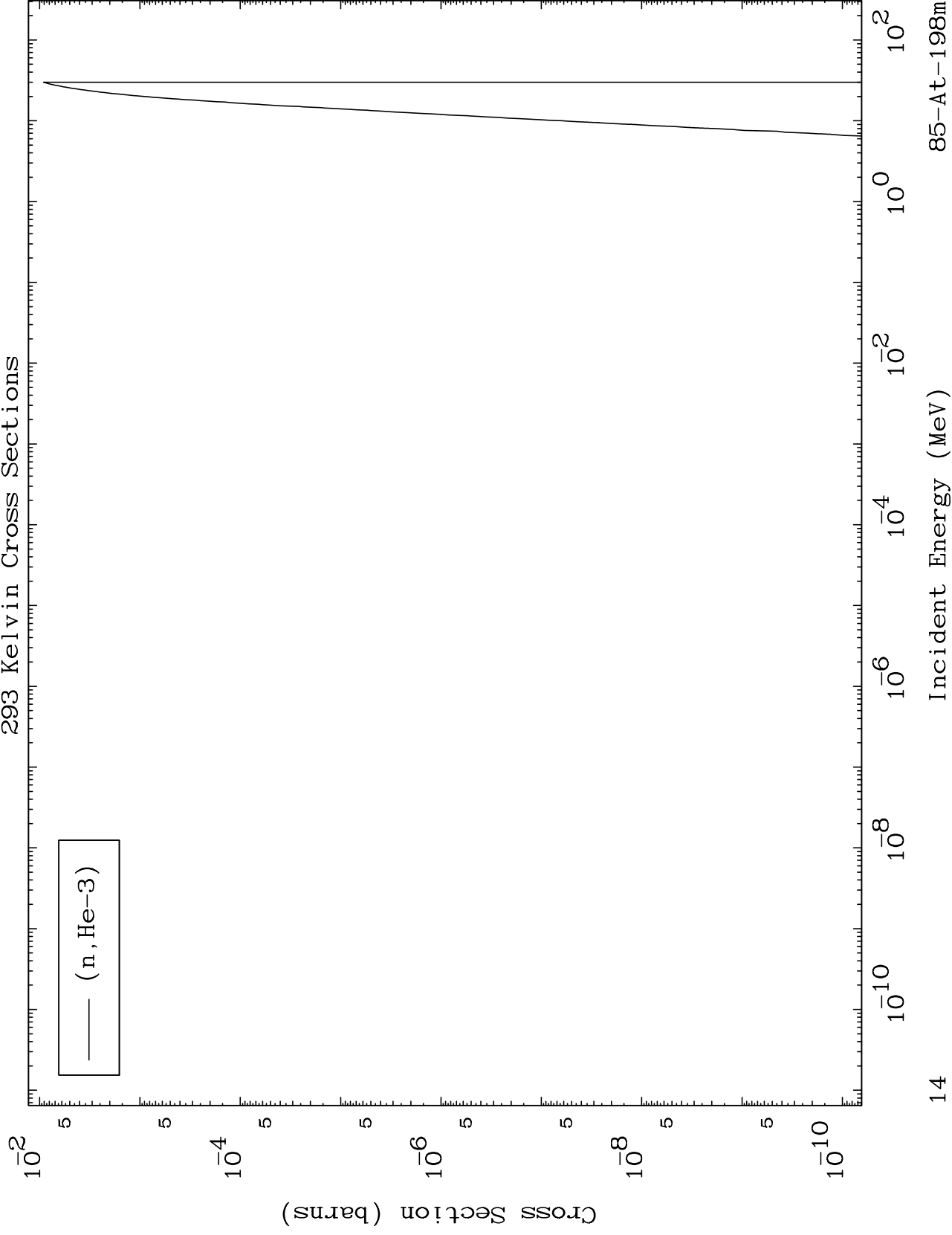
85-At-198m



MAT 8511

(n,He3) Levels  
293 Kelvin Cross Sections

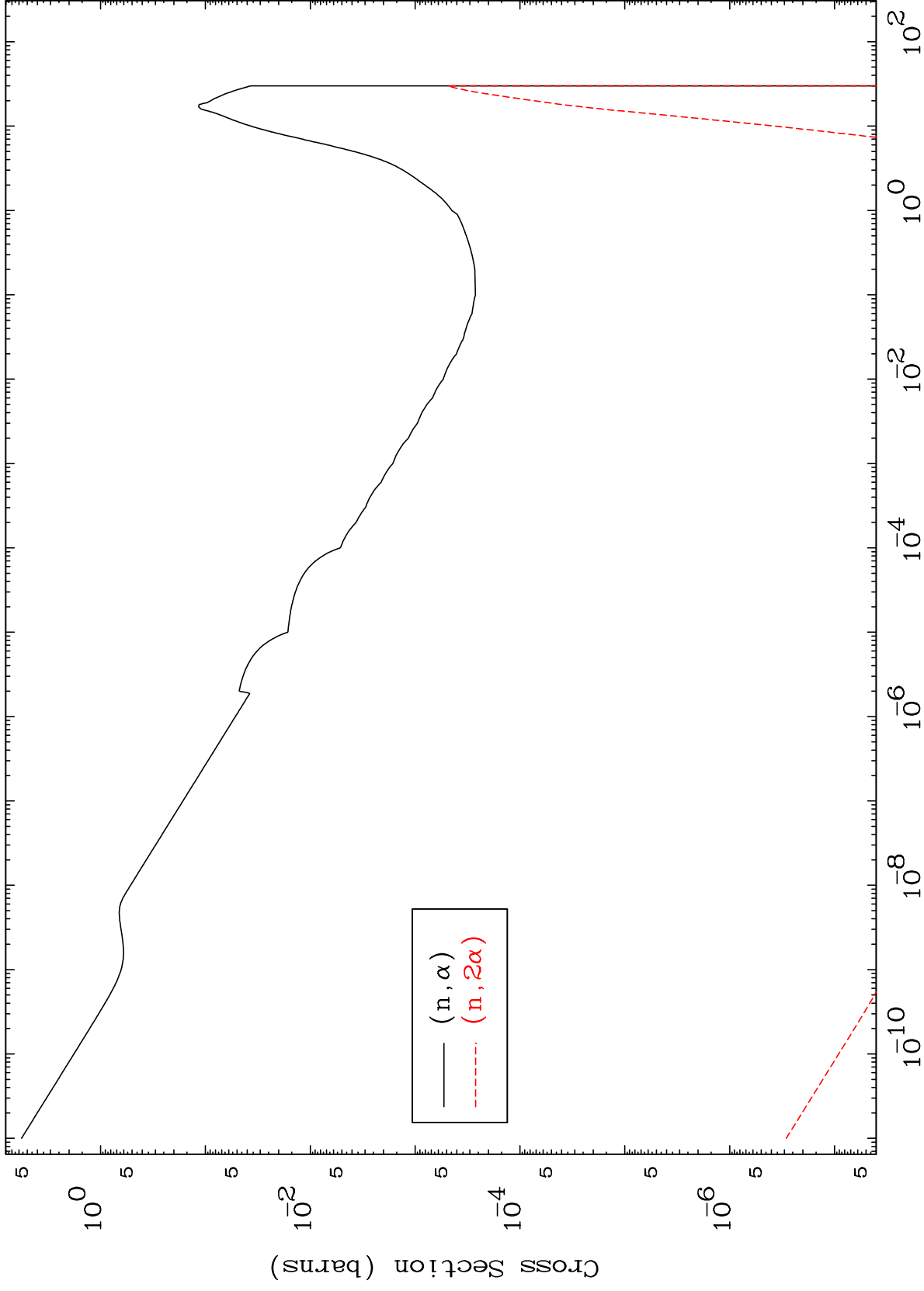
85-At-198m



MAT 8511

(n,α) Levels  
293 Kelvin Cross Sections

85-At-198m



15

Incident Energy (MeV)

85-At-198m





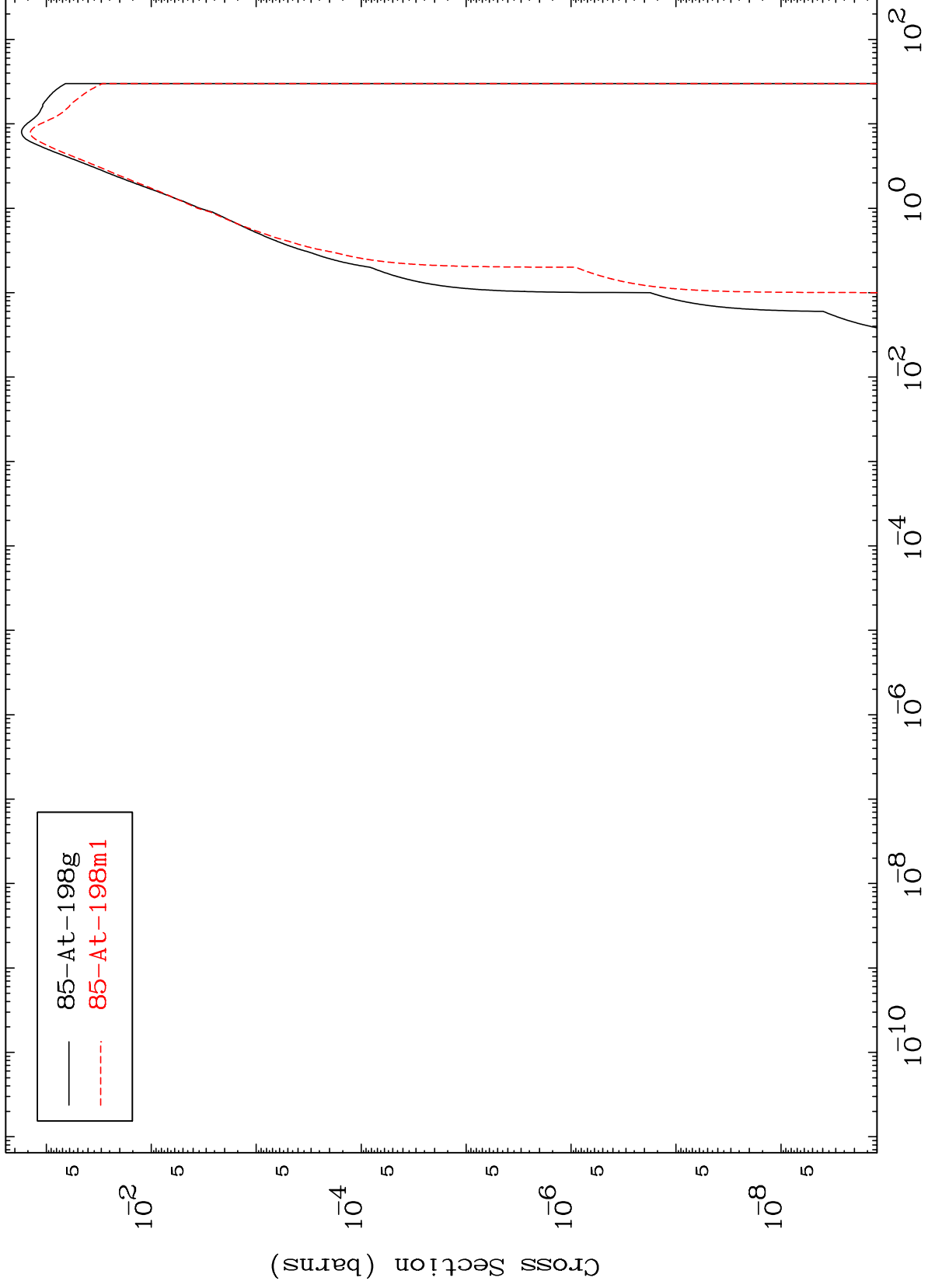




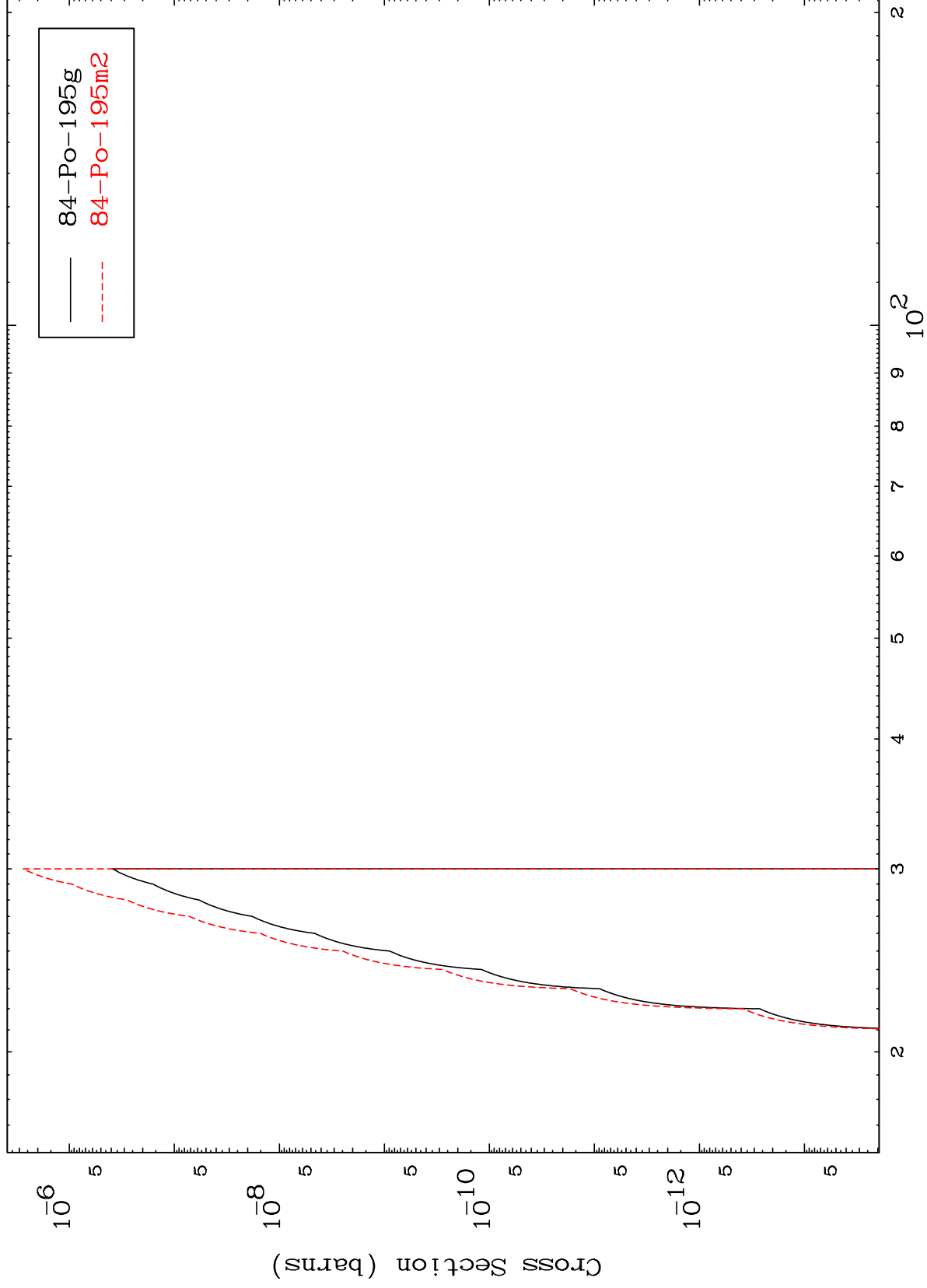
MAT 8511

Inelastic  
Radionuclide Production Cross Section

85-At-198m



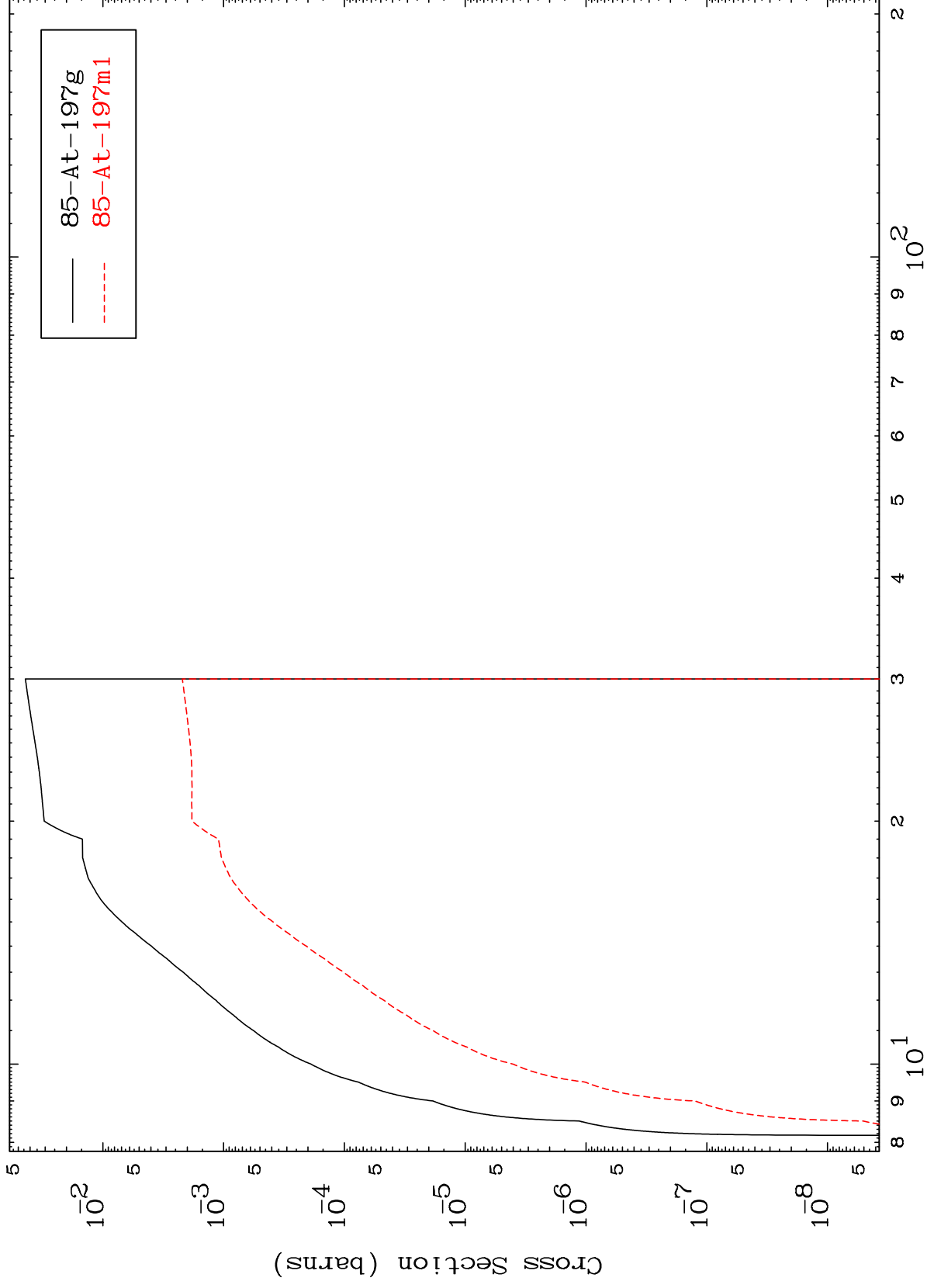
Radionuclide Production Cross Section



MAT 8511

85-At-198m

(n,2n)  
Radionuclide Production Cross Section



21

Incident Energy (MeV)

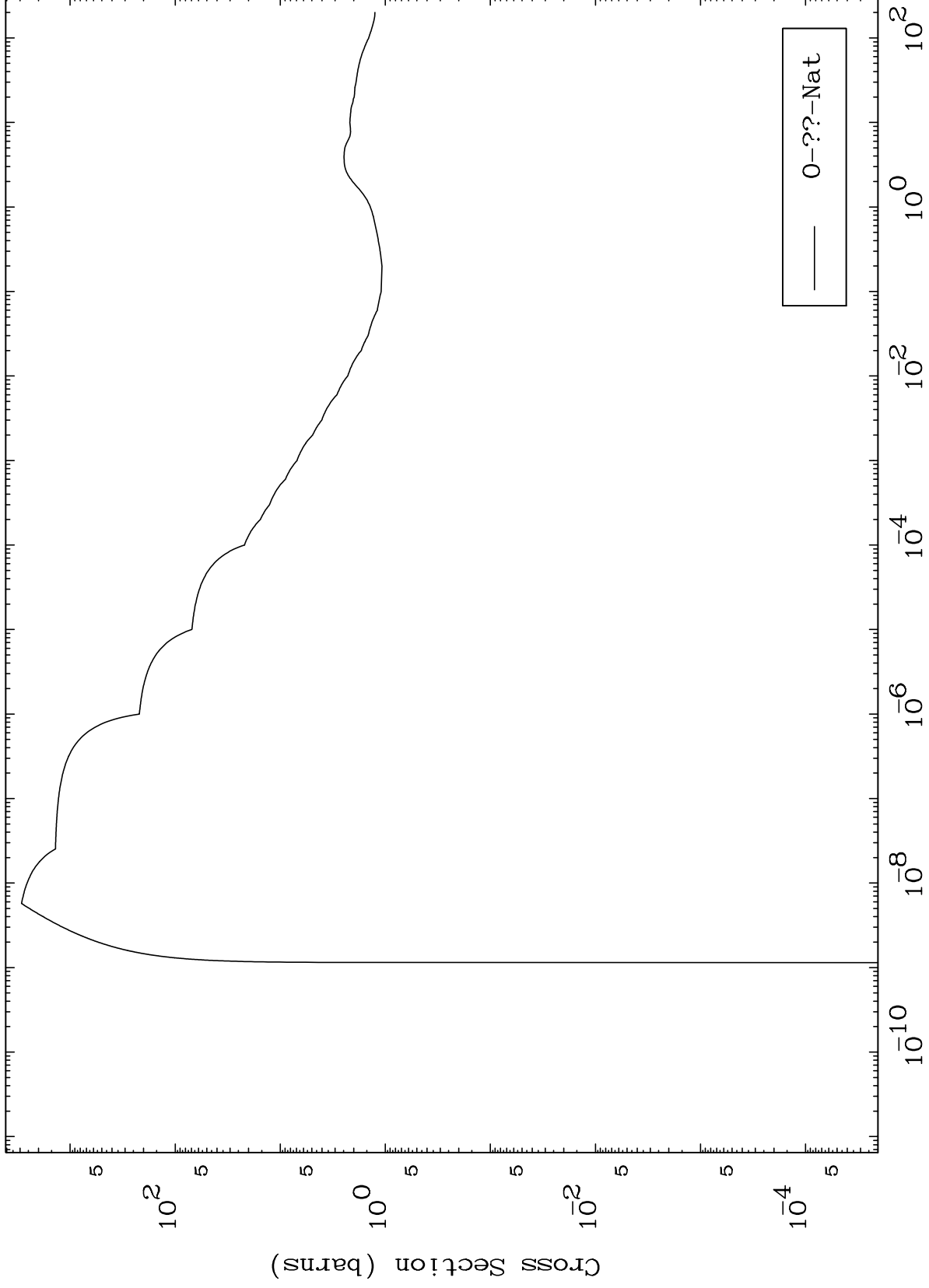
85-At-198m

MAT 8511

Fission

85-At-198m

Radionuclide Production Cross Section



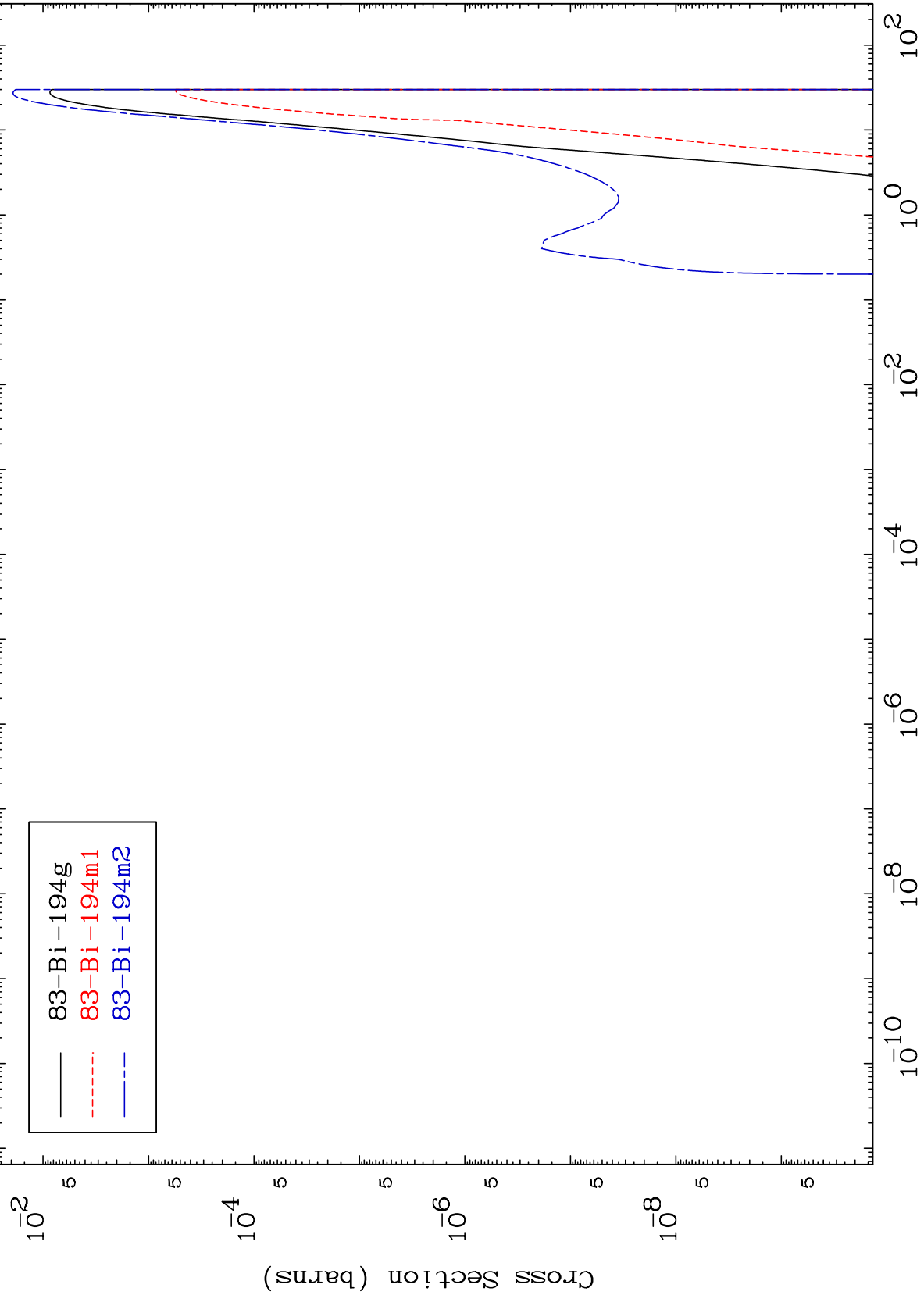
0-??-Nat

MAT 8511

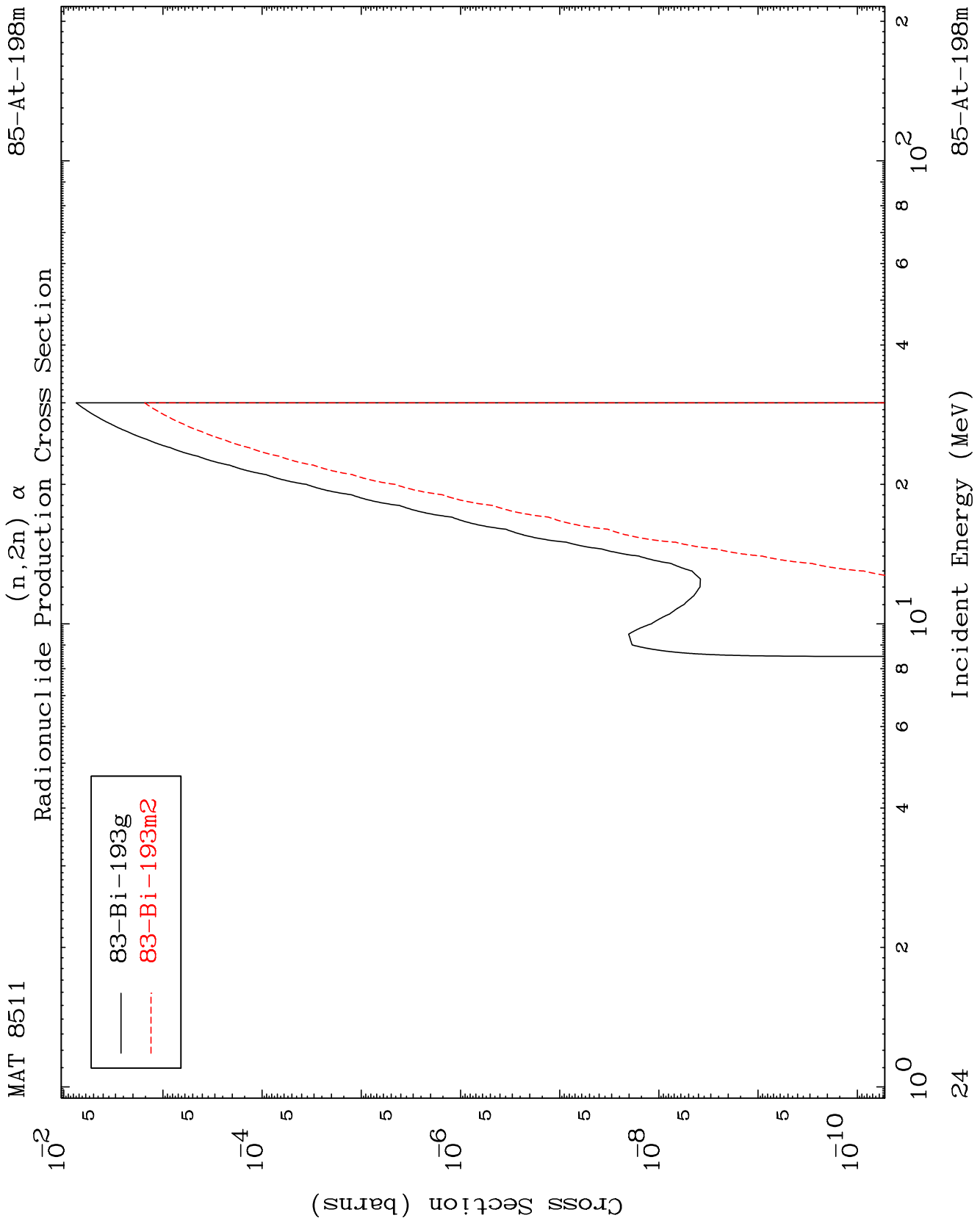
$(n, n') \alpha$

85-At-198m

Radionuclide Production Cross Section





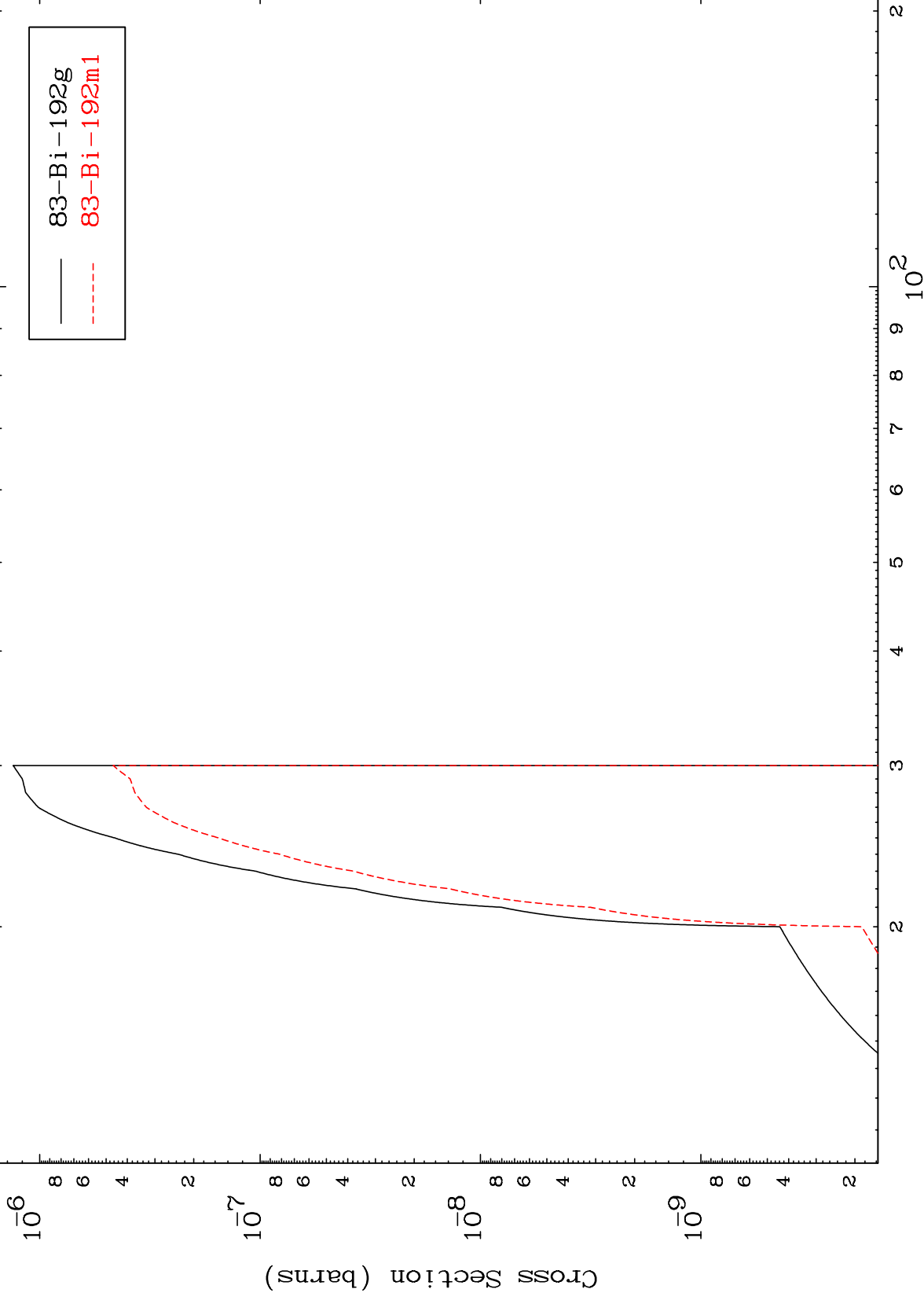


MAT 8511

(n,3n)  $\alpha$

85-At-198m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

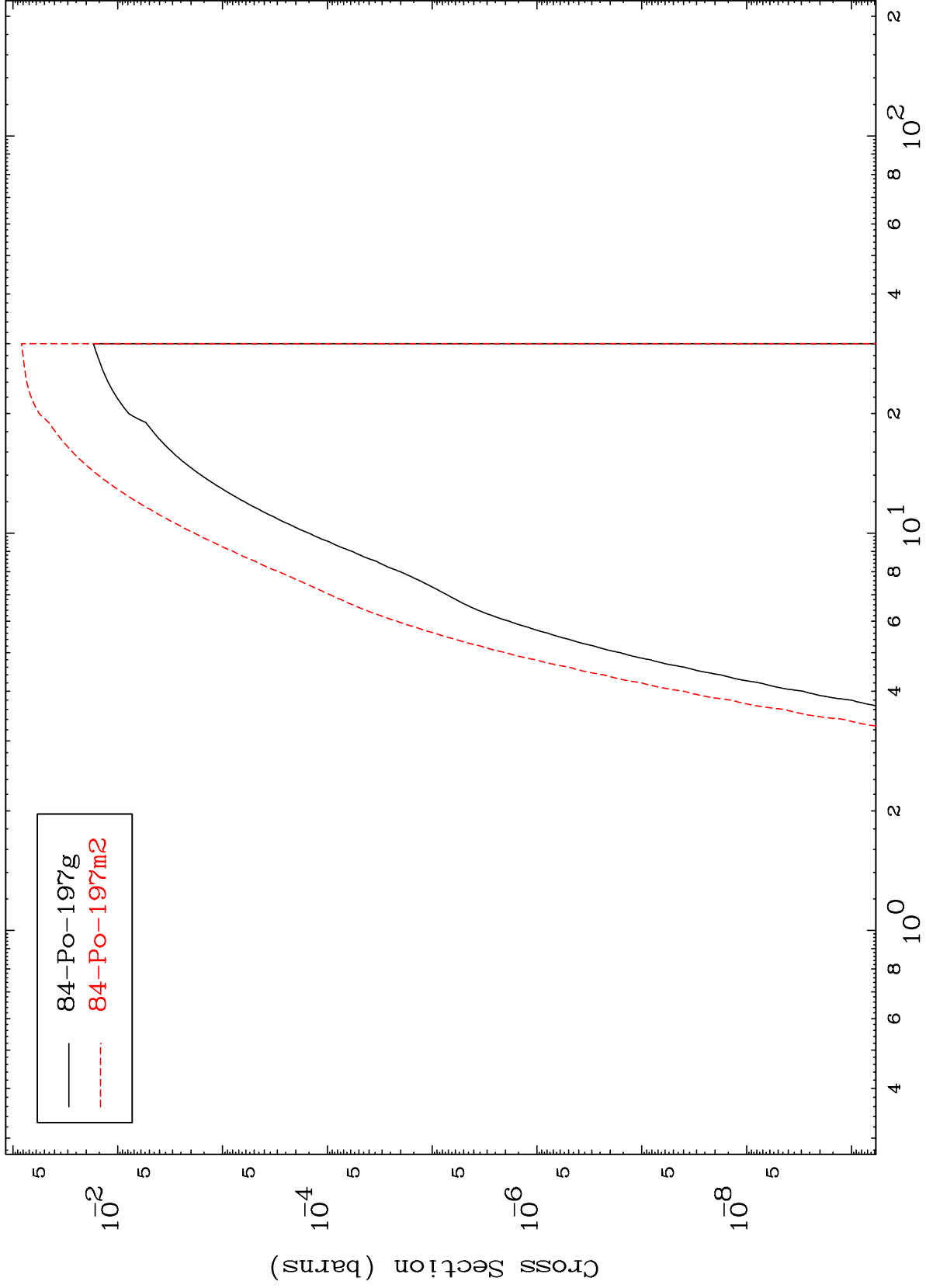
85-At-198m

MAT 8511

(n,n') p

85-At-198m

Radionuclide Production Cross Section



26

Incident Energy (MeV)

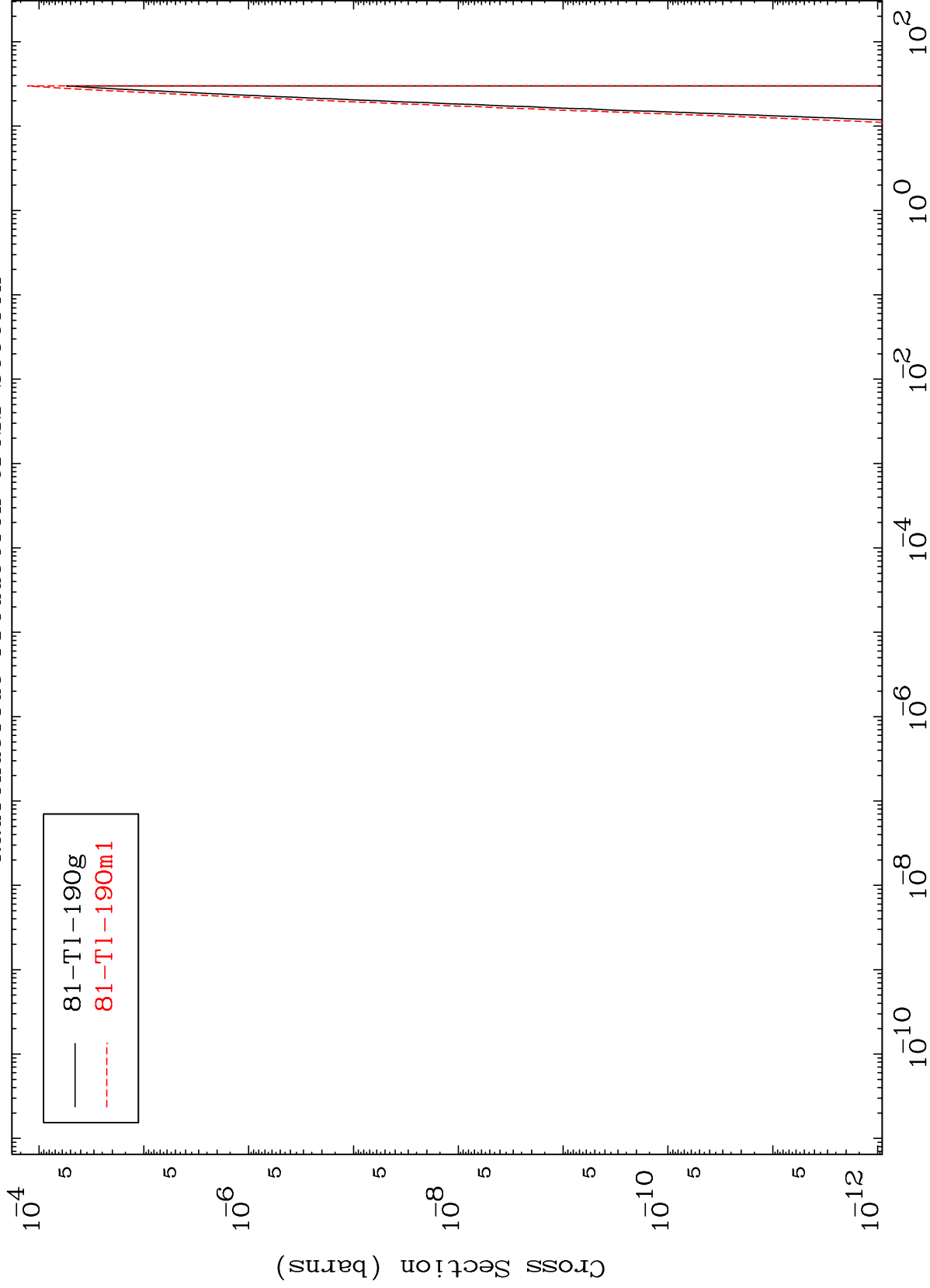
85-At-198m

MAT 8511

(n,n') 2α

85-At-198m

Radionuclide Production Cross Section

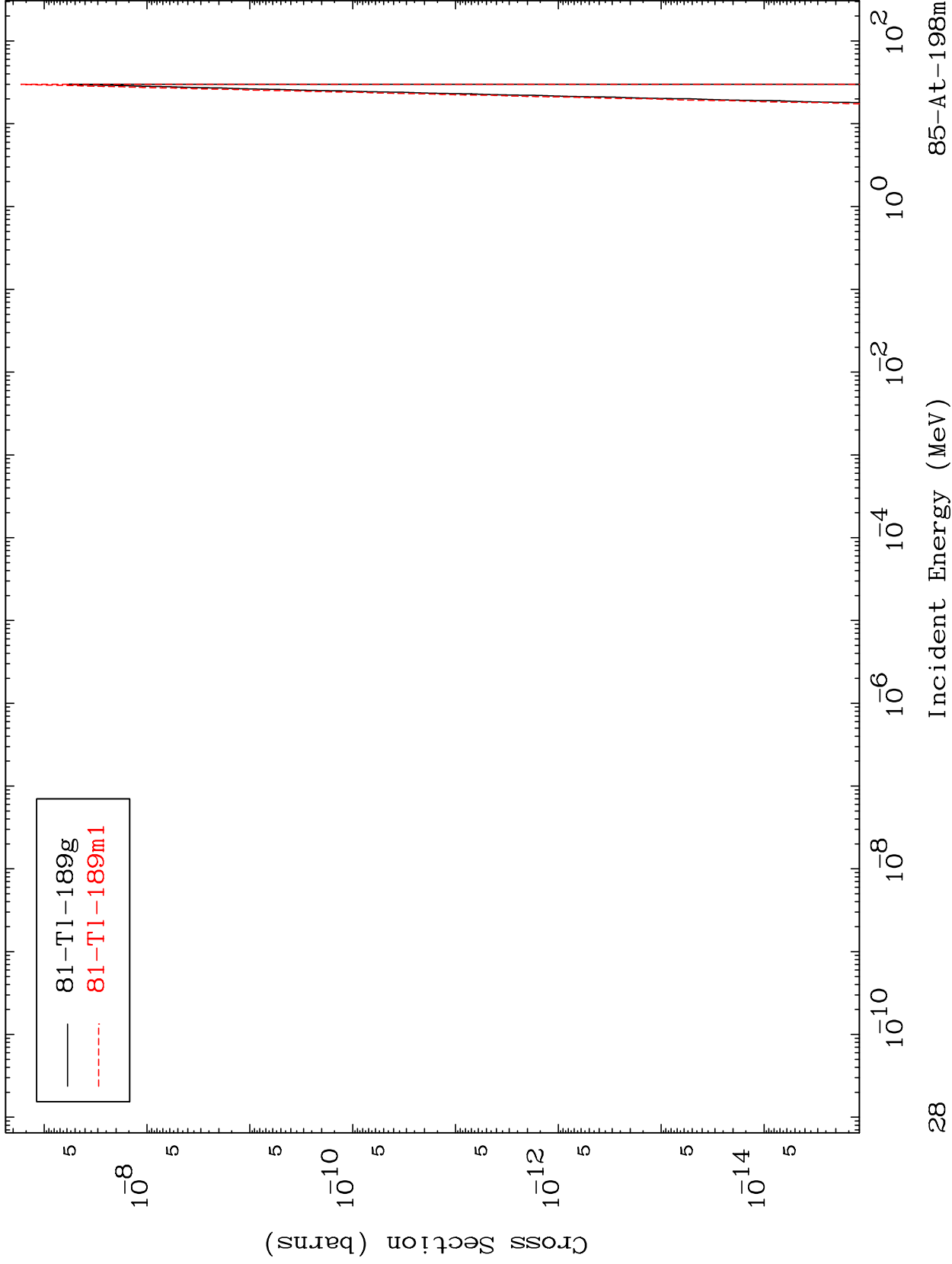


MAT 8511

(n,2n) 2α

85-At-198m

Radionuclide Production Cross Section

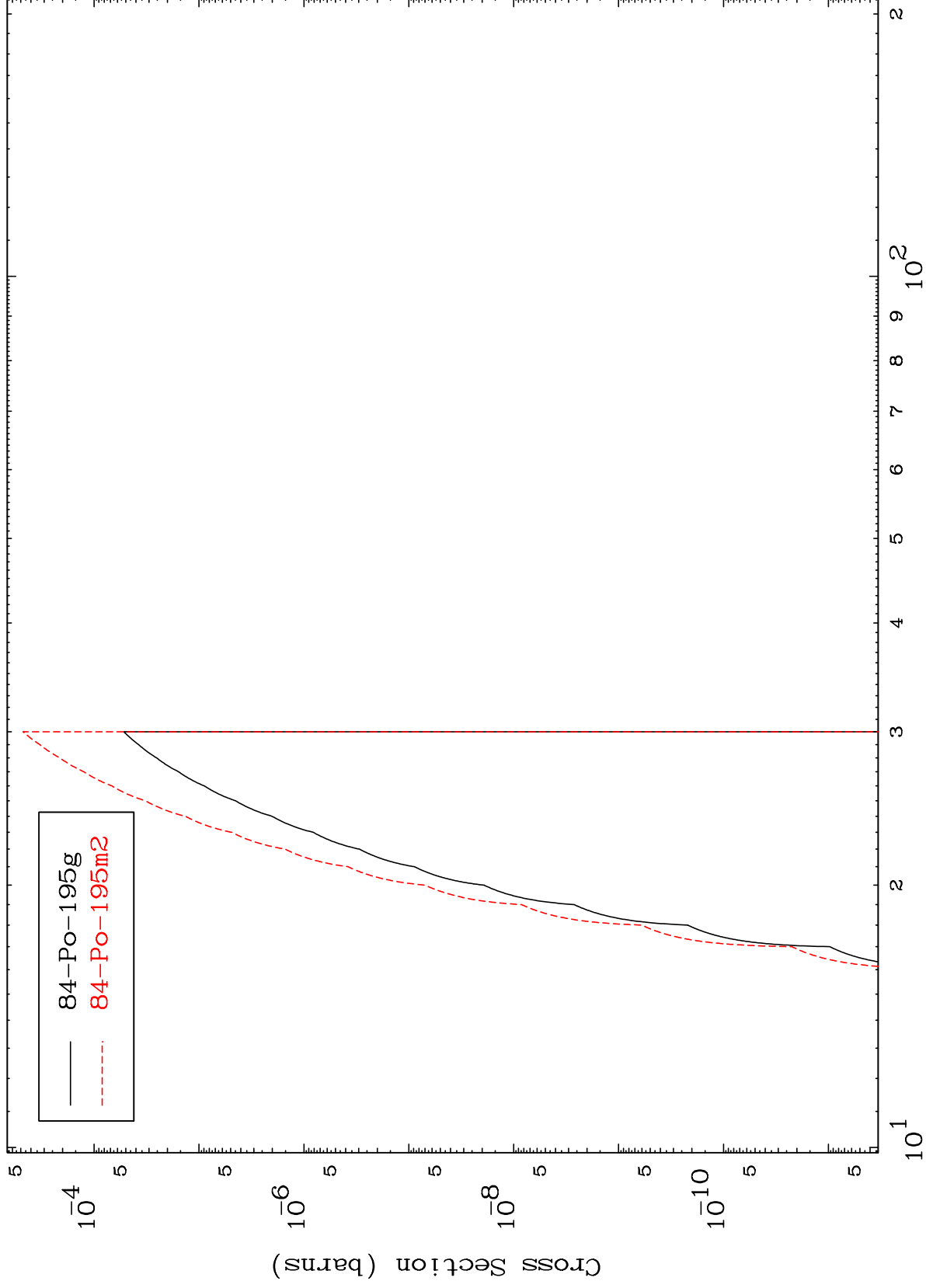


MAT 8511

(n,n') t

85-At-198m

Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-198m

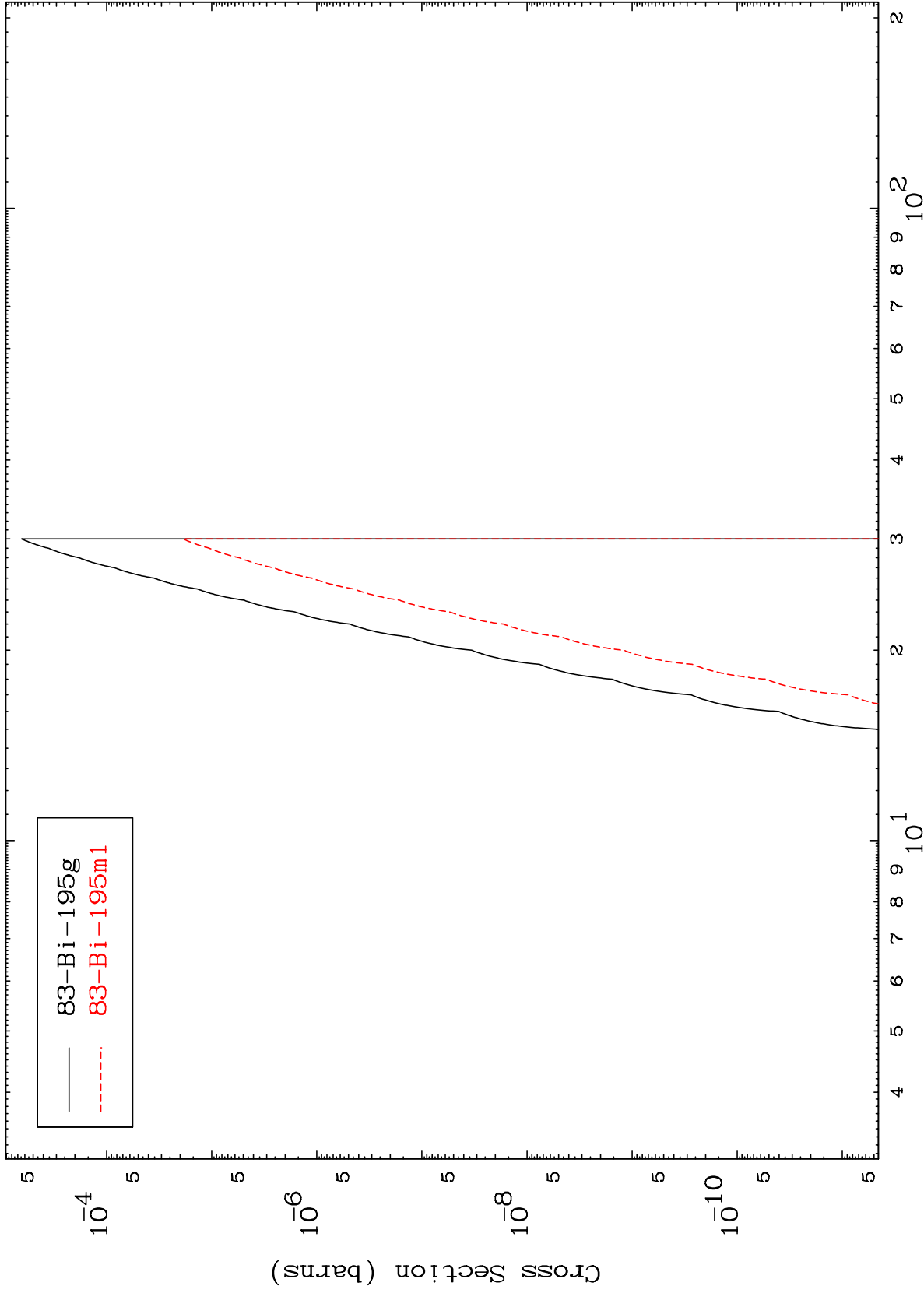
29

MAT 8511

(n,n') He-3

85-At-198m

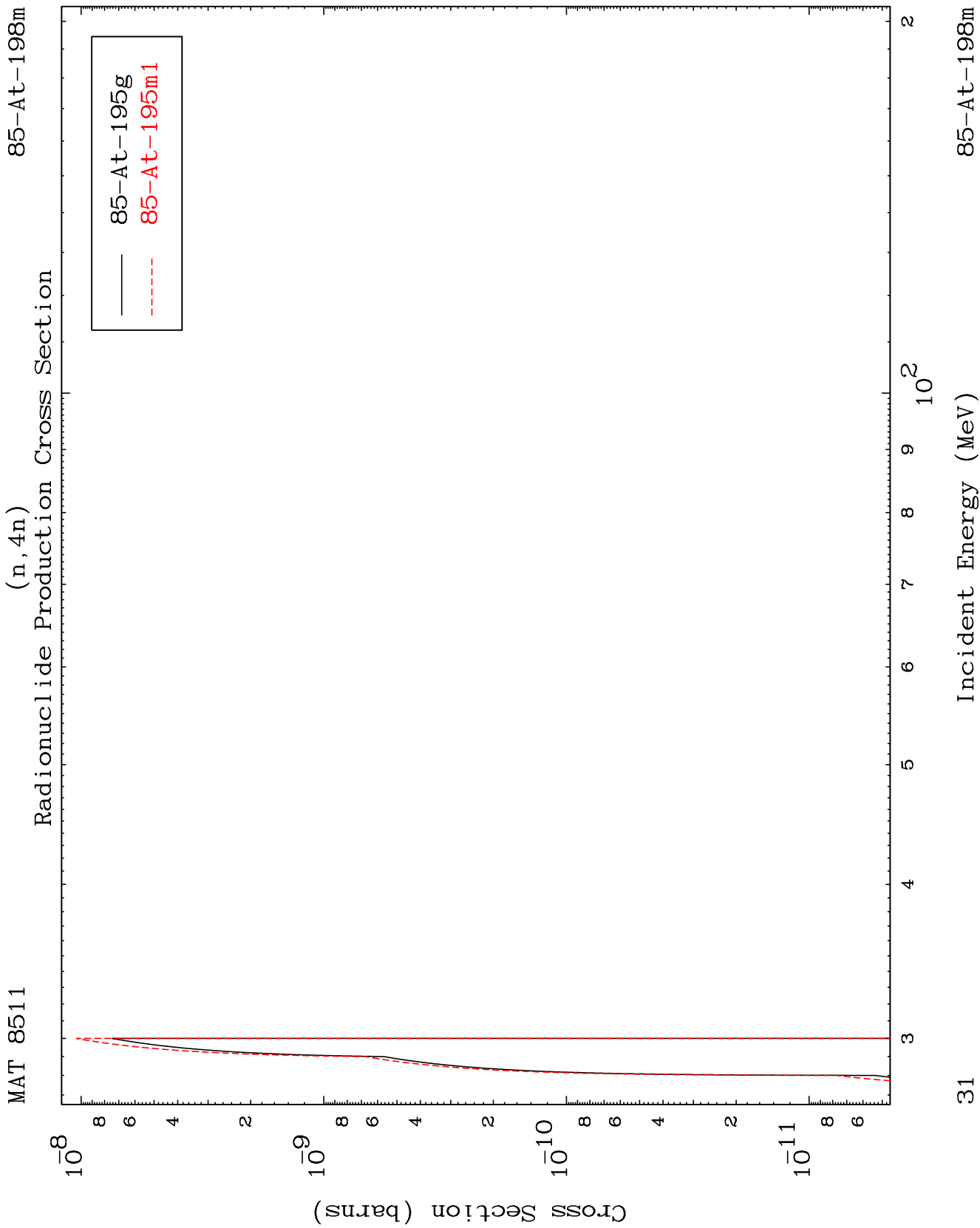
Radionuclide Production Cross Section



30

Incident Energy (MeV)

85-At-198m



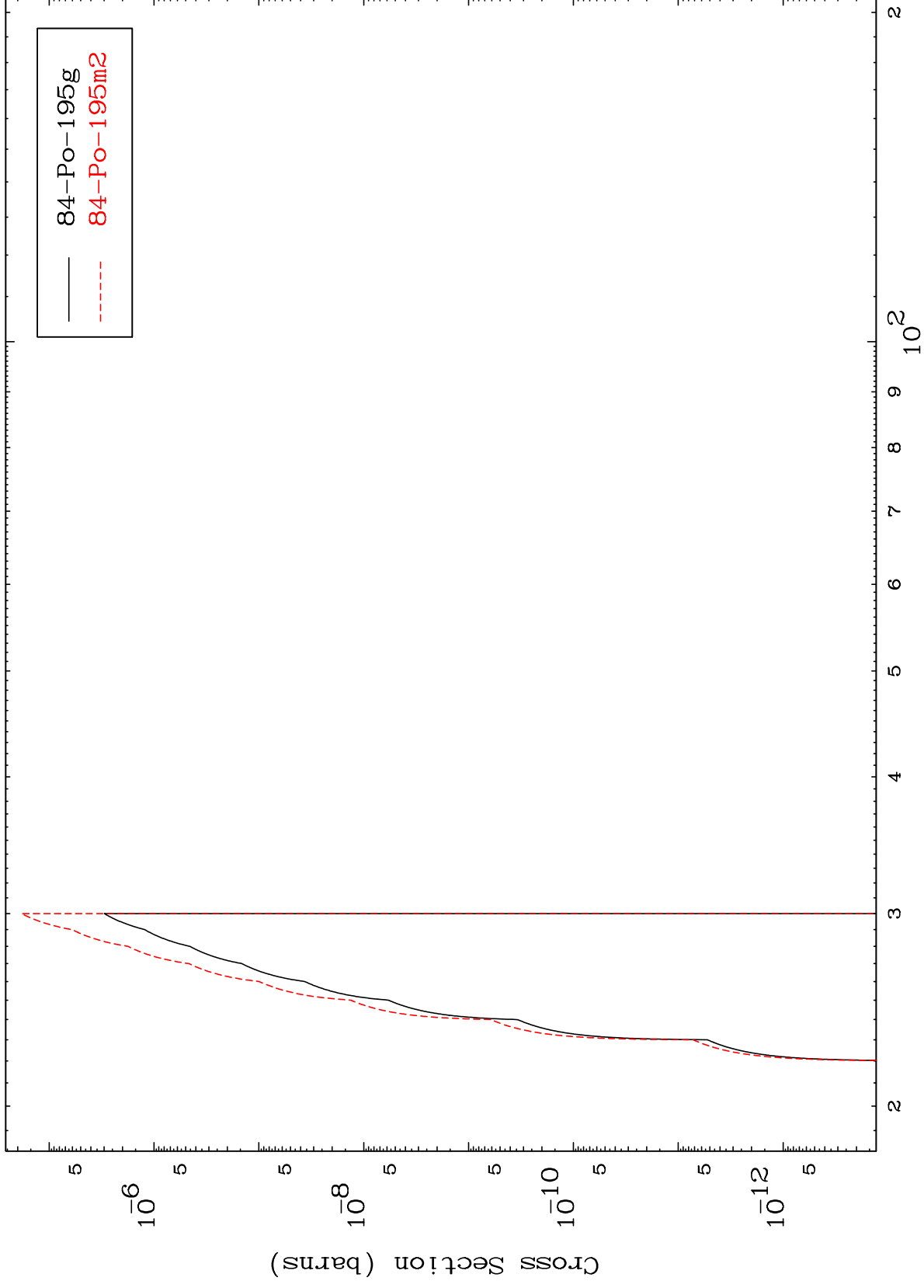


MAT 8511

(n,3n) p

85-At-198m

Radionuclide Production Cross Section



32

Incident Energy (MeV)

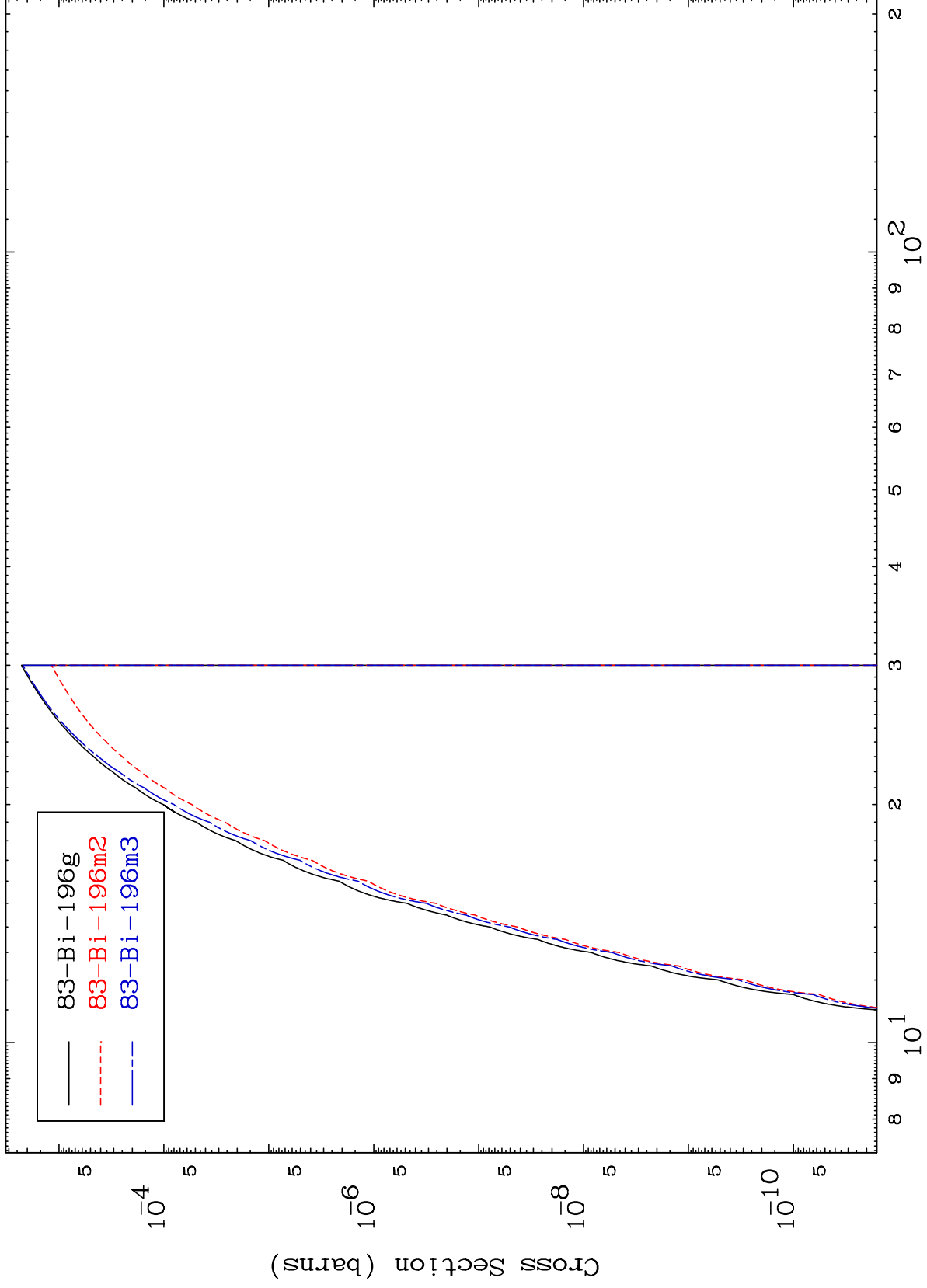
85-At-198m

MAT 8511

(n,2n) p

85-At-198m

Radionuclide Production Cross Section



33

Incident Energy (MeV)

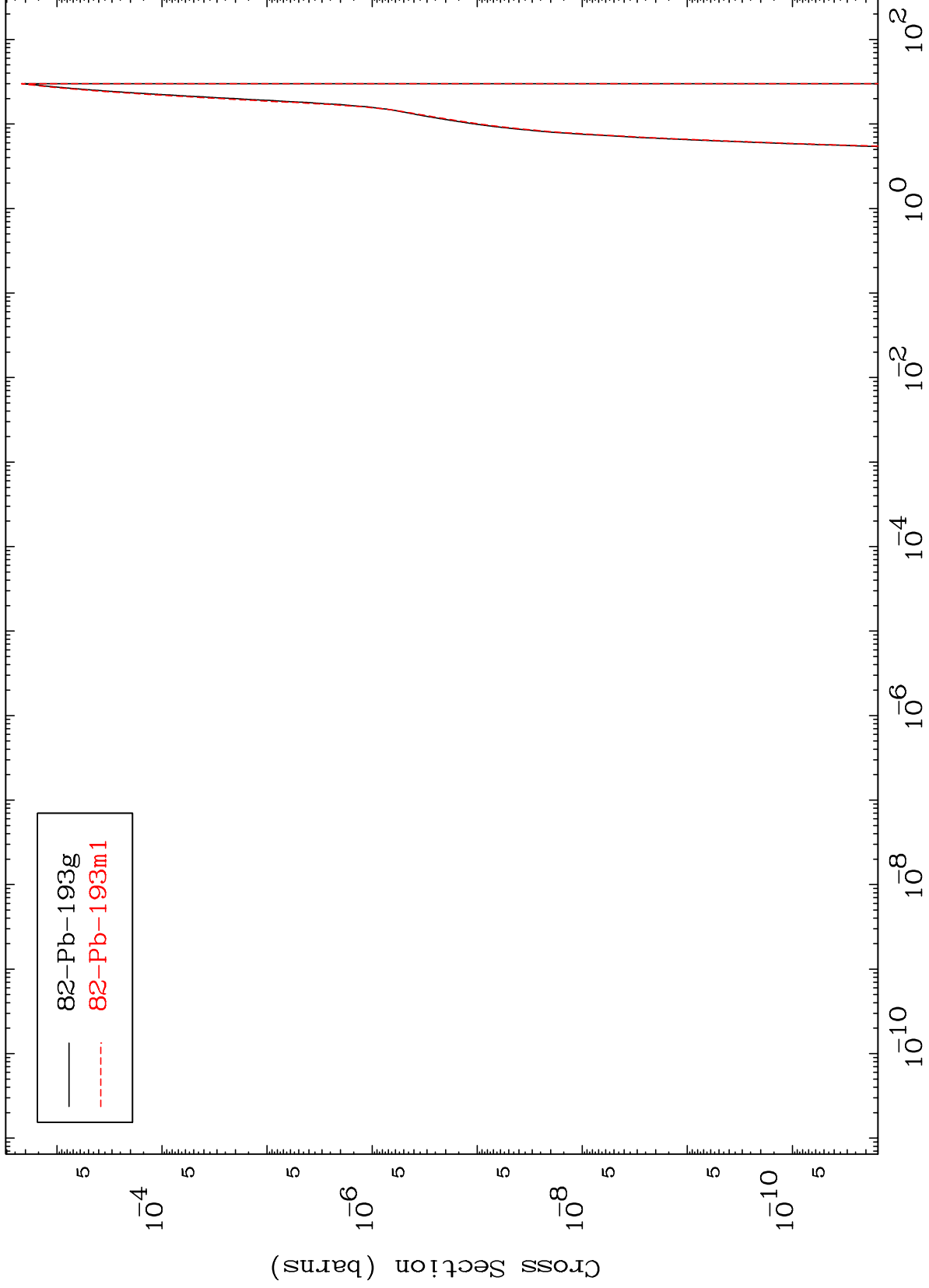
85-At-198m

MAT 8511

(n,n') p  $\alpha$

85-At-198m

Radionuclide Production Cross Section

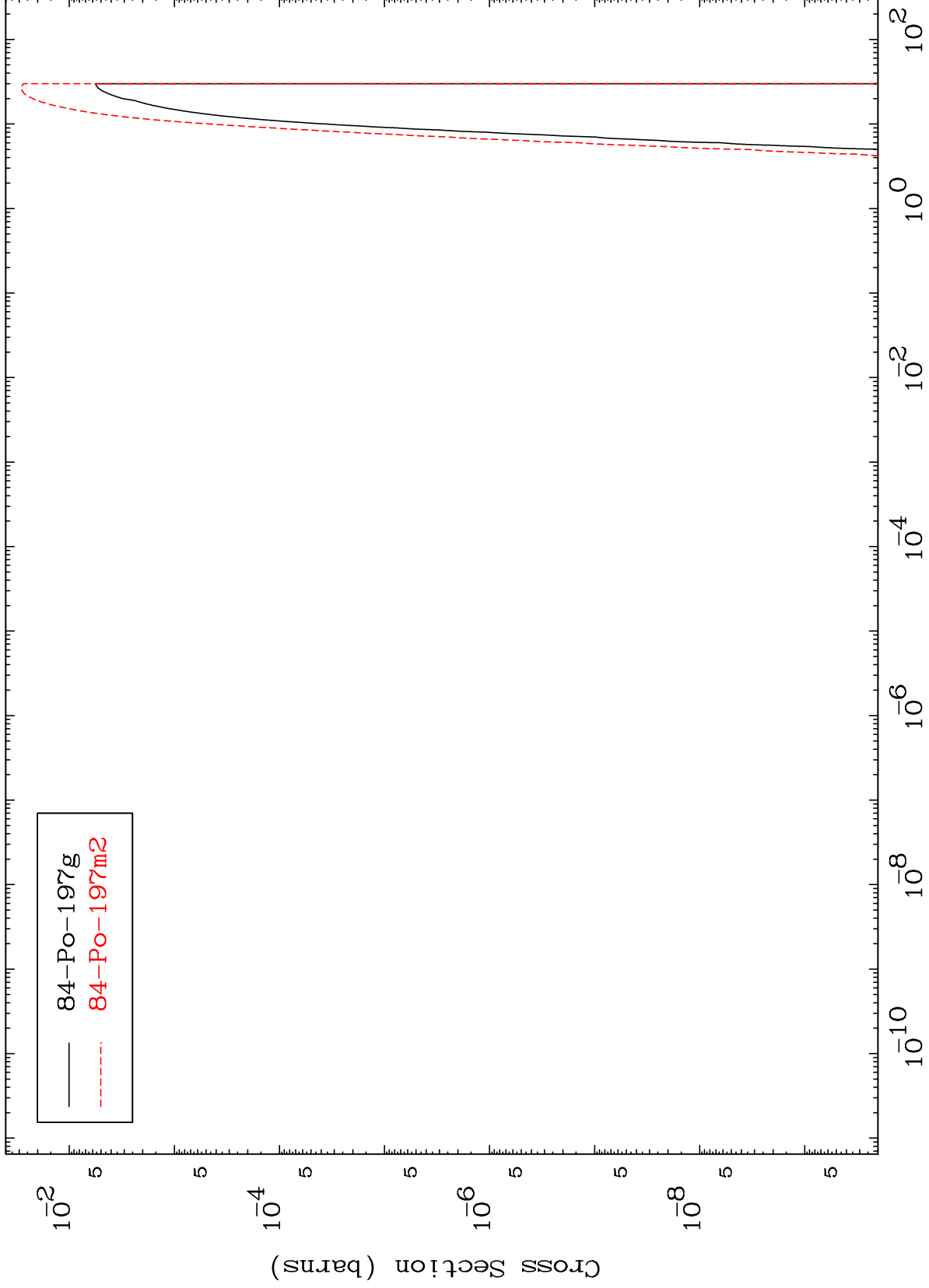
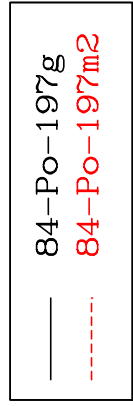


MAT 8511

(n,d)

85-At-198m

Radionuclide Production Cross Section



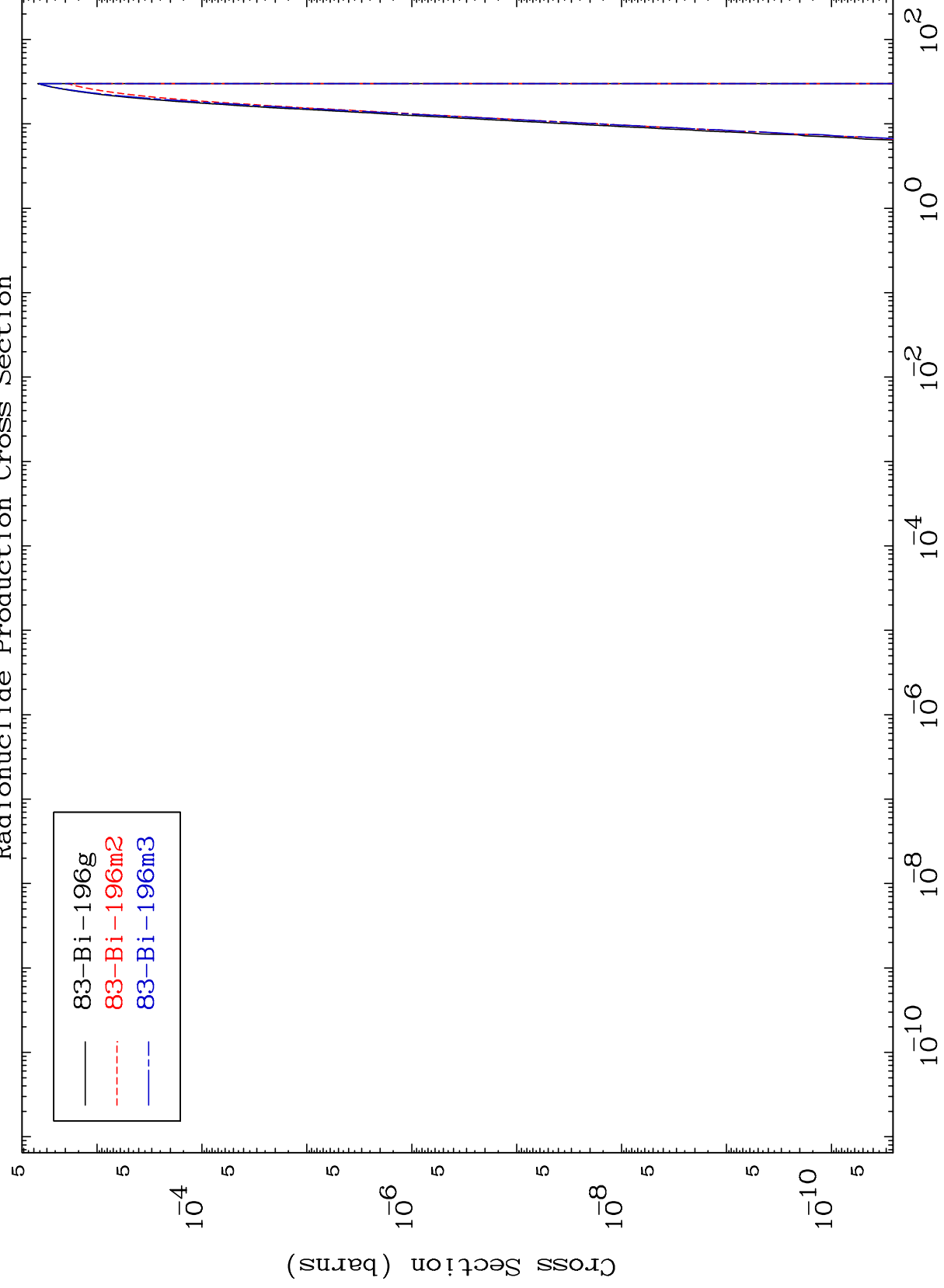
35

85-At-198m

MAT 8511

85-At-198m

(n,He-3)  
Radionuclide Production Cross Section



36

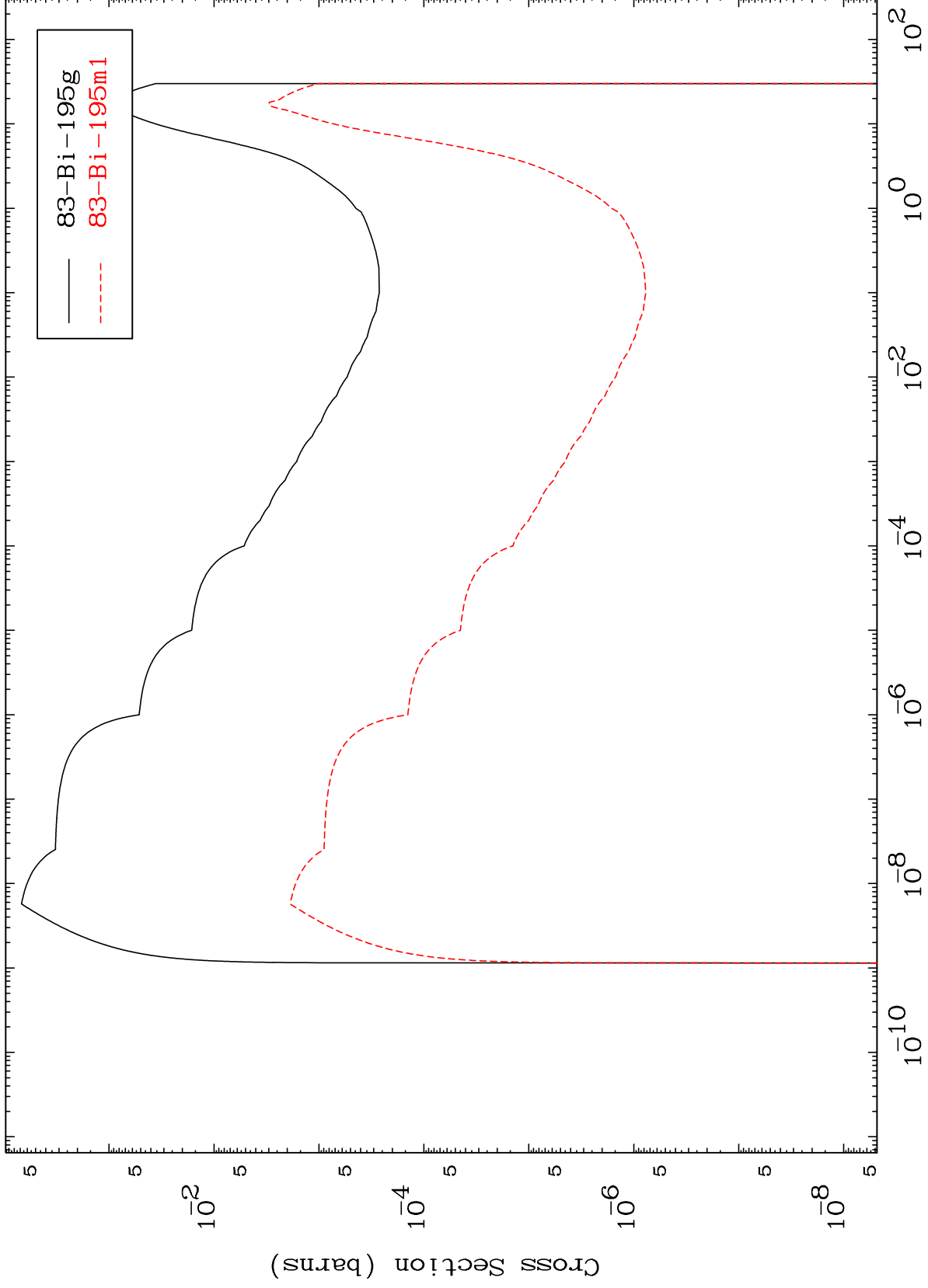
85-At-198m

Incident Energy (MeV)

MAT 8511

85-At-198m

(n,  $\alpha$ )  
Radionuclide Production Cross Section



37

85-At-198m

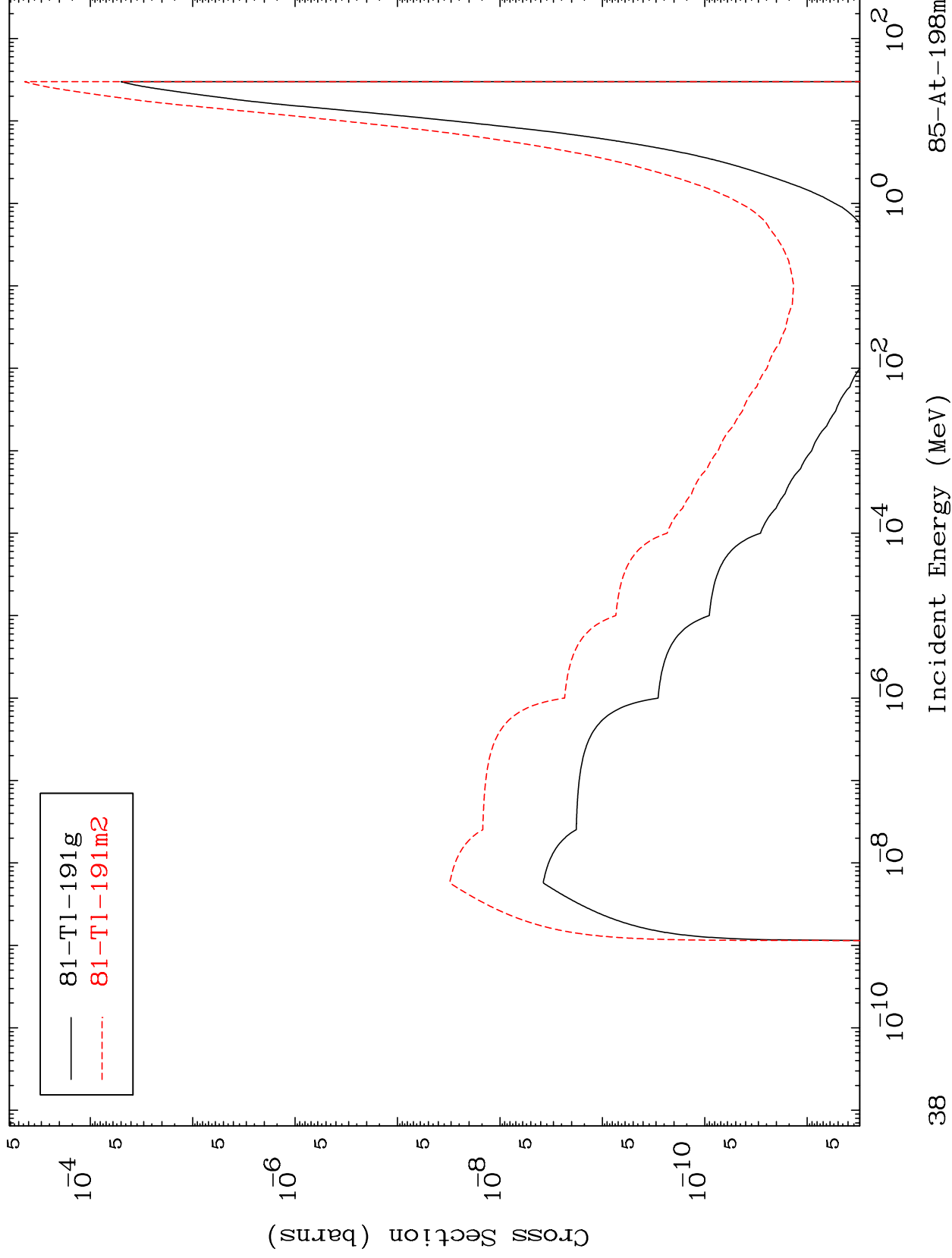
Incident Energy (MeV)

MAT 8511

(n,2α)

85-At-198m

Radionuclide Production Cross Section

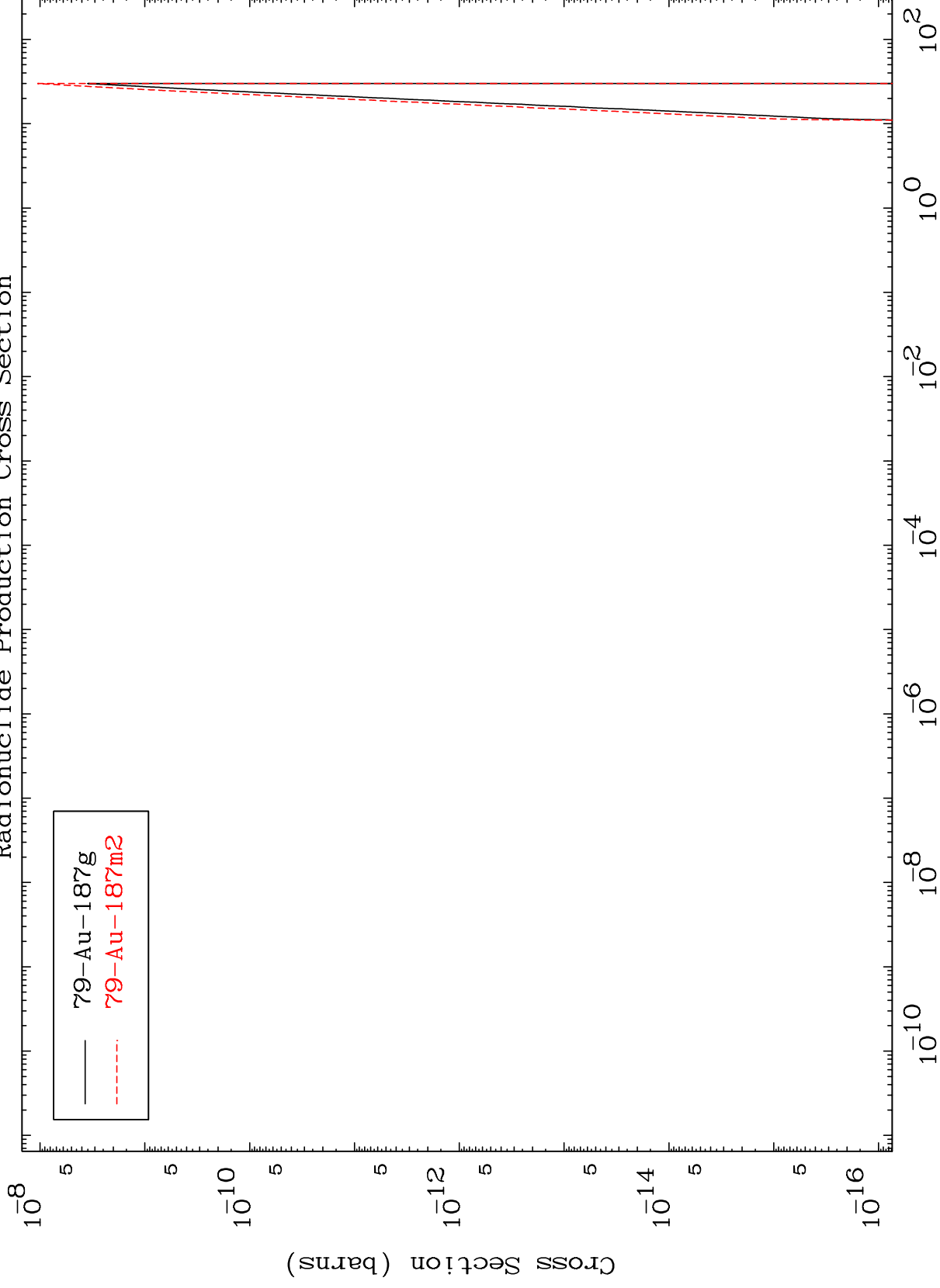


MAT 8511

(n,3 $\alpha$ )

85-At-198m

Radionuclide Production Cross Section



39

Incident Energy (MeV)

85-At-198m

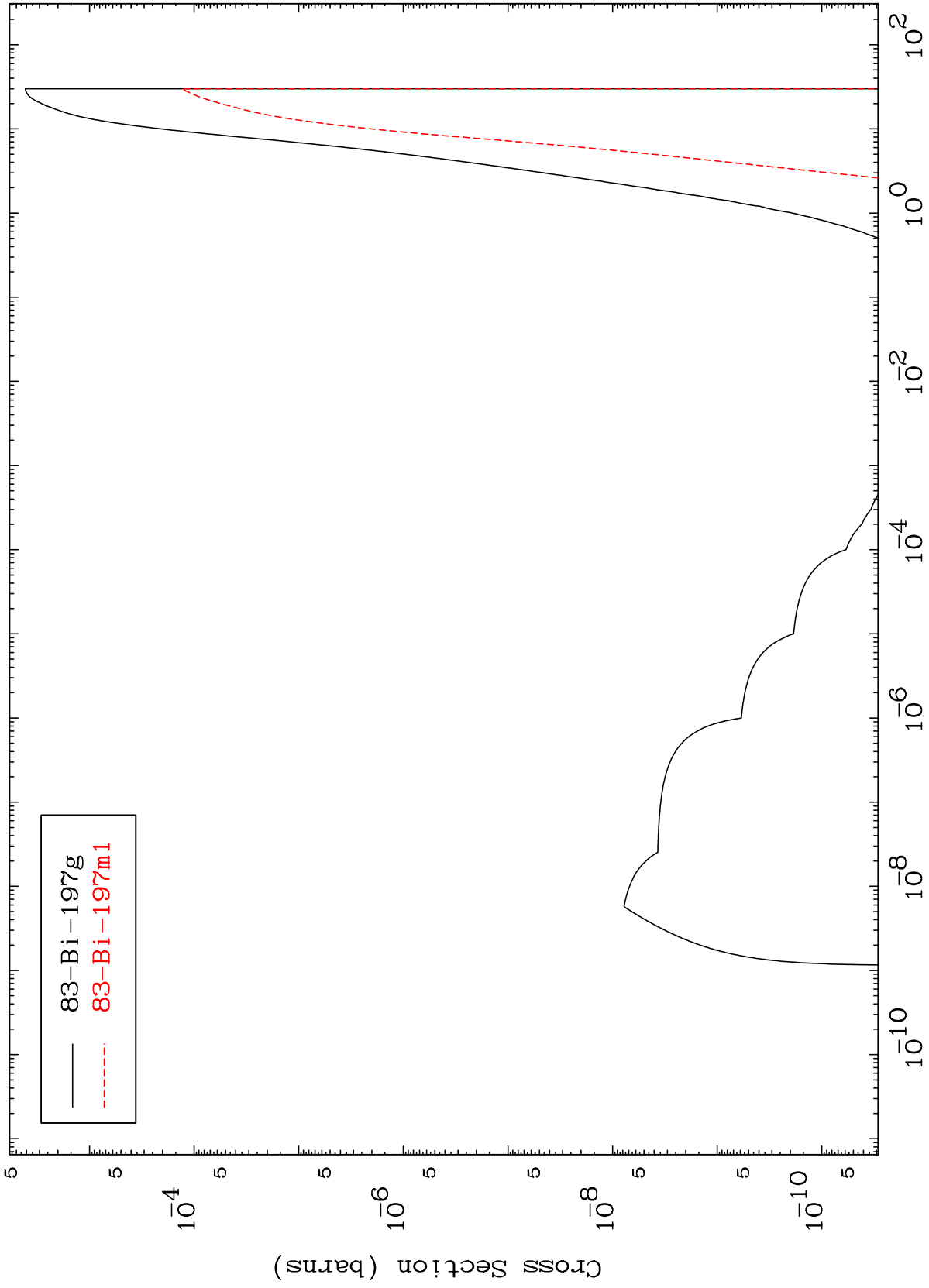


MAT 8511

(n,2p)

85-At-198m

Radionuclide Production Cross Section



40

Incident Energy (MeV)

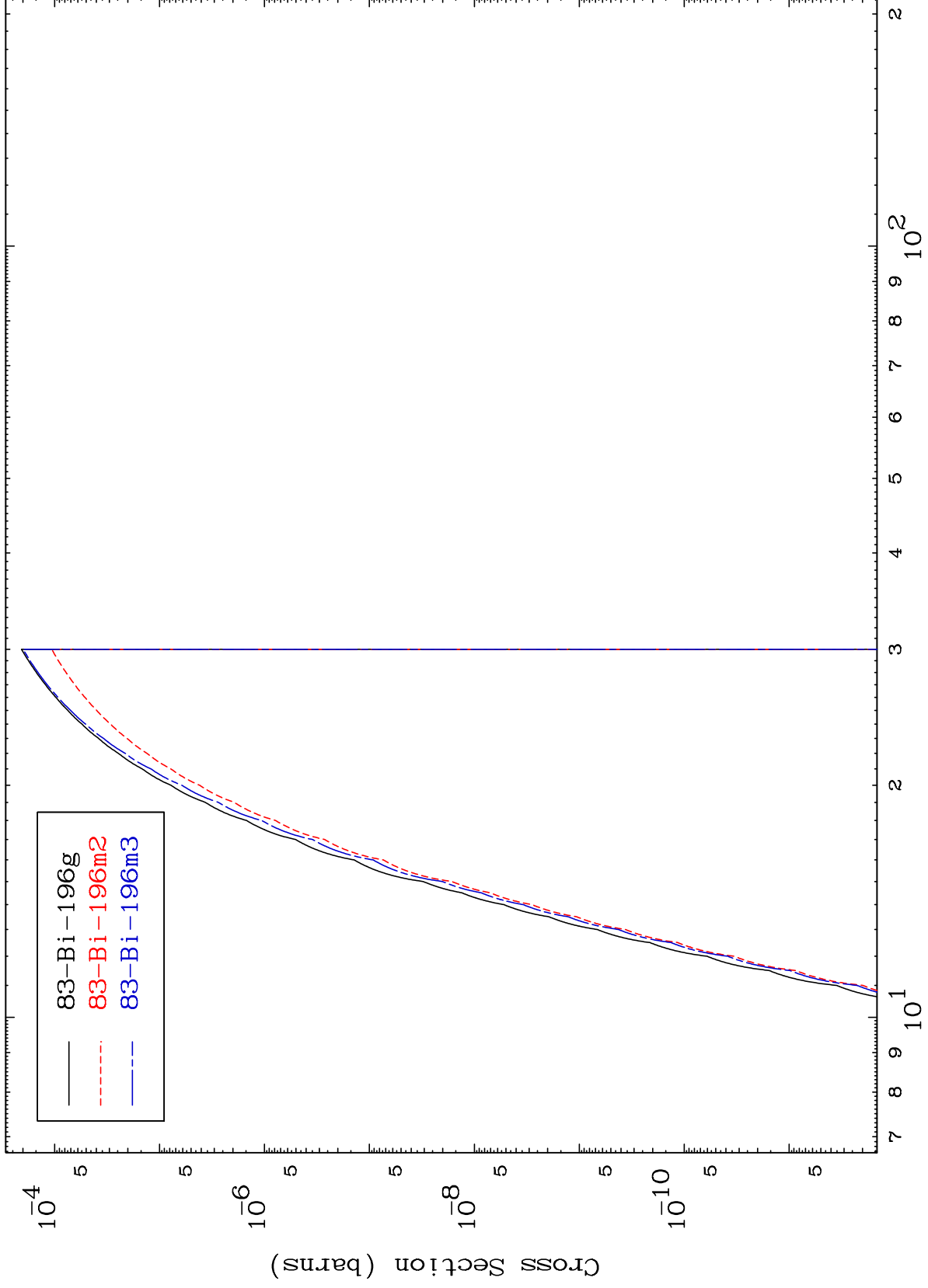
85-At-198m

MAT 8511

(n,p) d

85-At-198m

Radionuclide Production Cross Section



41

Incident Energy (MeV)

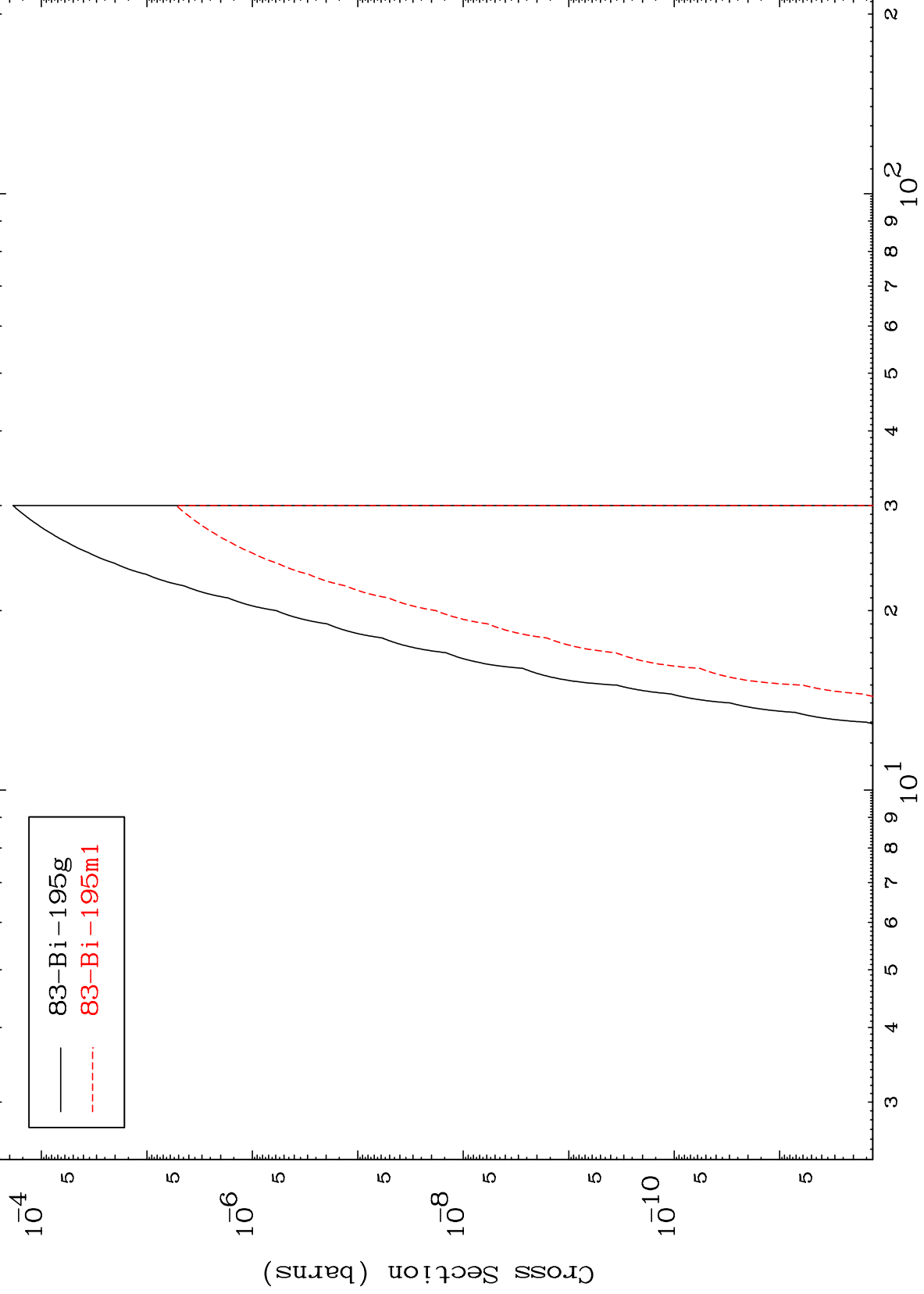
85-At-198m

MAT 8511

(n,p) t

85-At-198m

Radionuclide Production Cross Section



83-Bi-195g  
83-Bi-195m1

Incident Energy (MeV)

85-At-198m

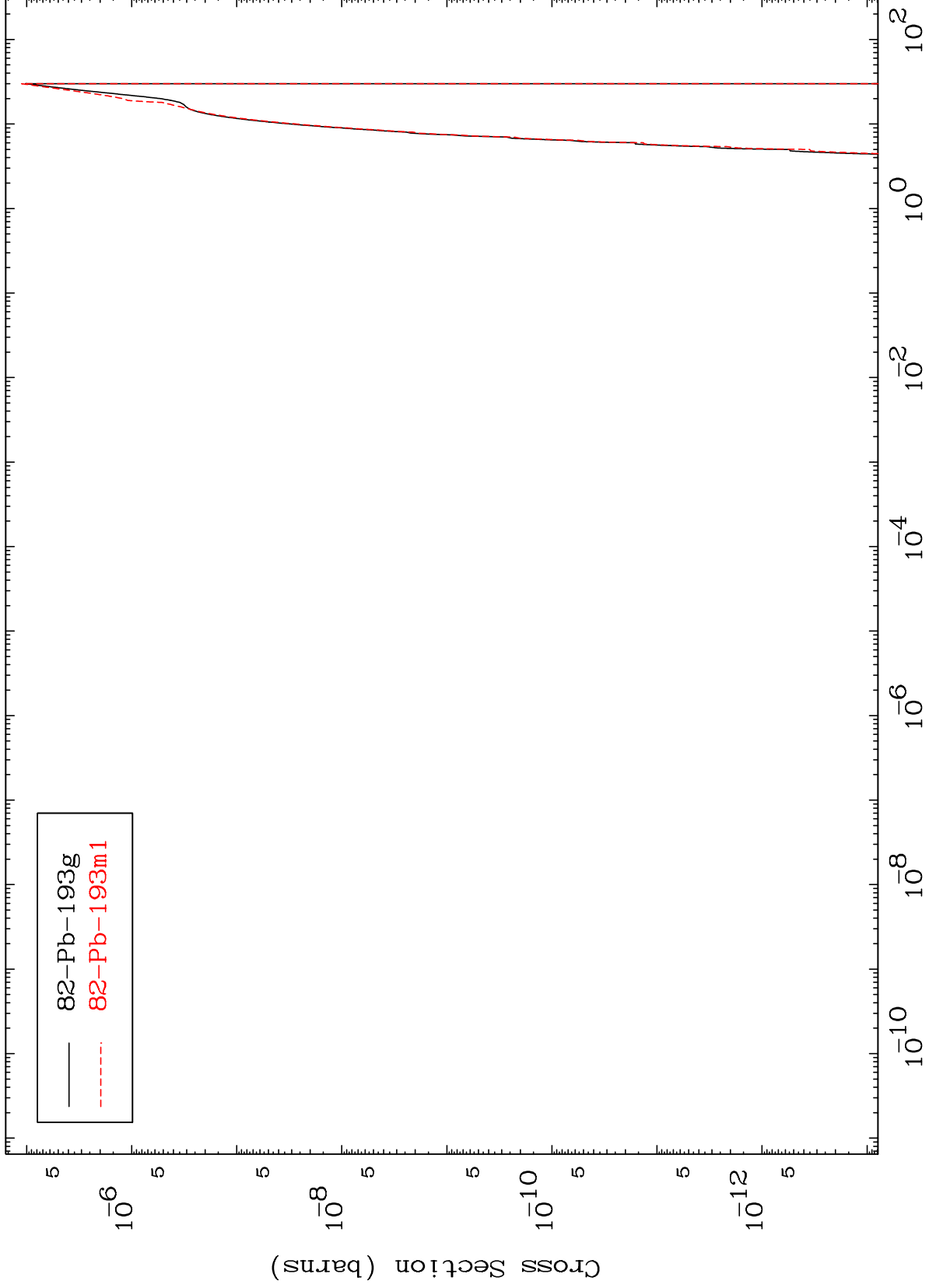
42

MAT 8511

(n,d)  $\alpha$

85-At-198m

Radionuclide Production Cross Section



43

Incident Energy (MeV)

85-At-198m