

Program EVALPLOT
(Version 2021-1)

by

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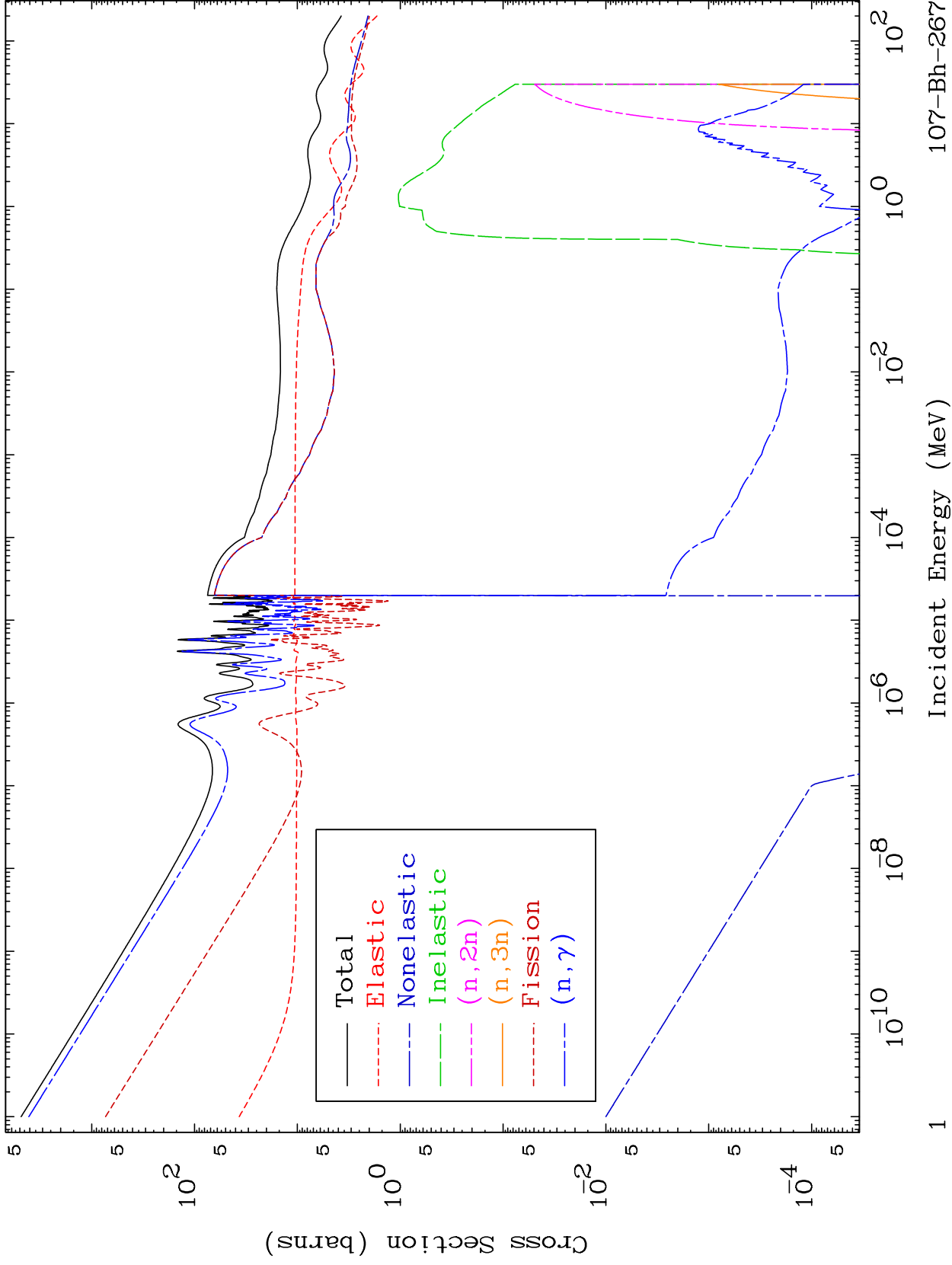
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Press Mouse Button to Start

MAT 767

Neutron Major
293 Kelvin Cross Sections

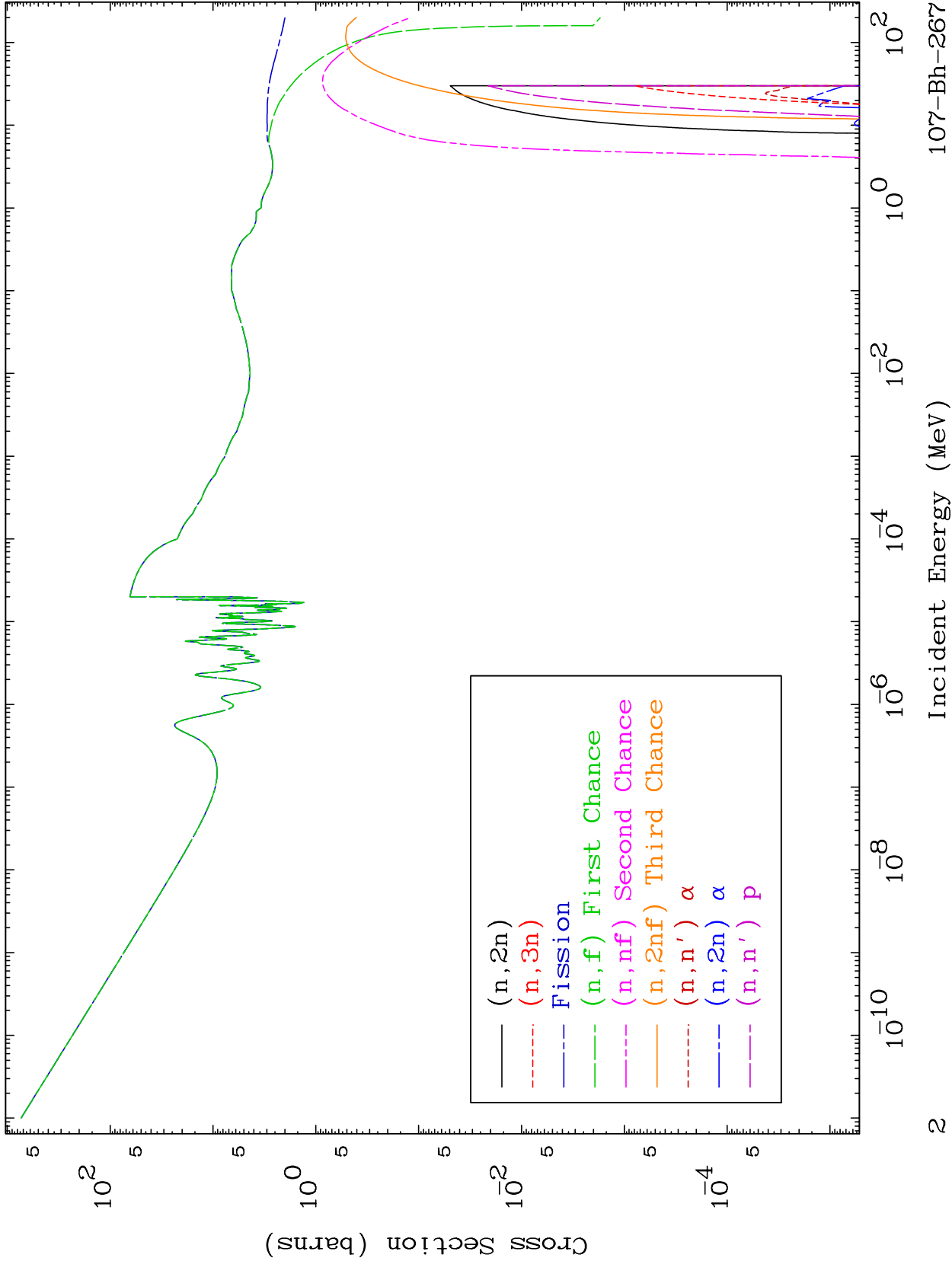
107-Bh-267

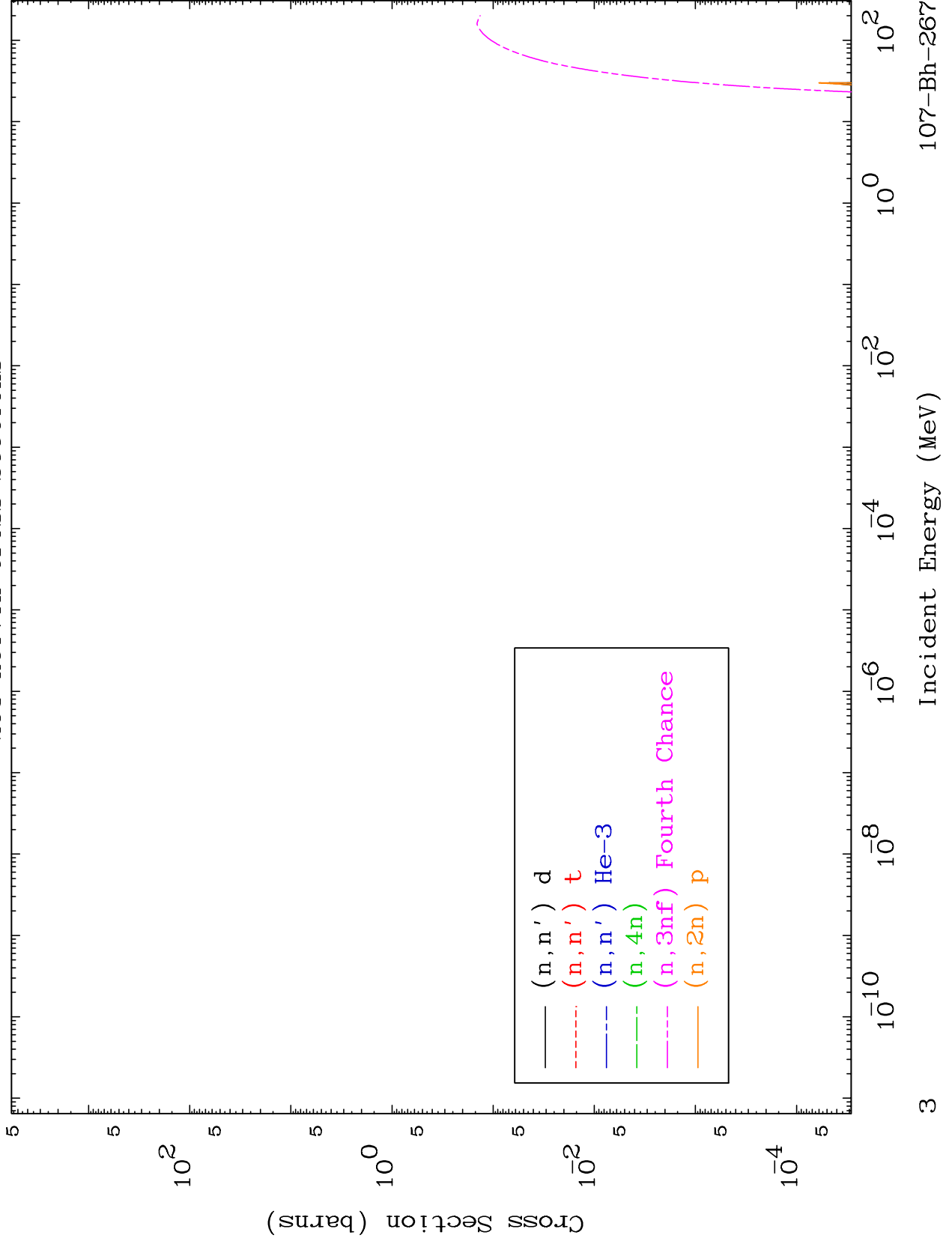


MAT 767

Neutron Absorption
293 Kelvin Cross Sections

107-Bh-267

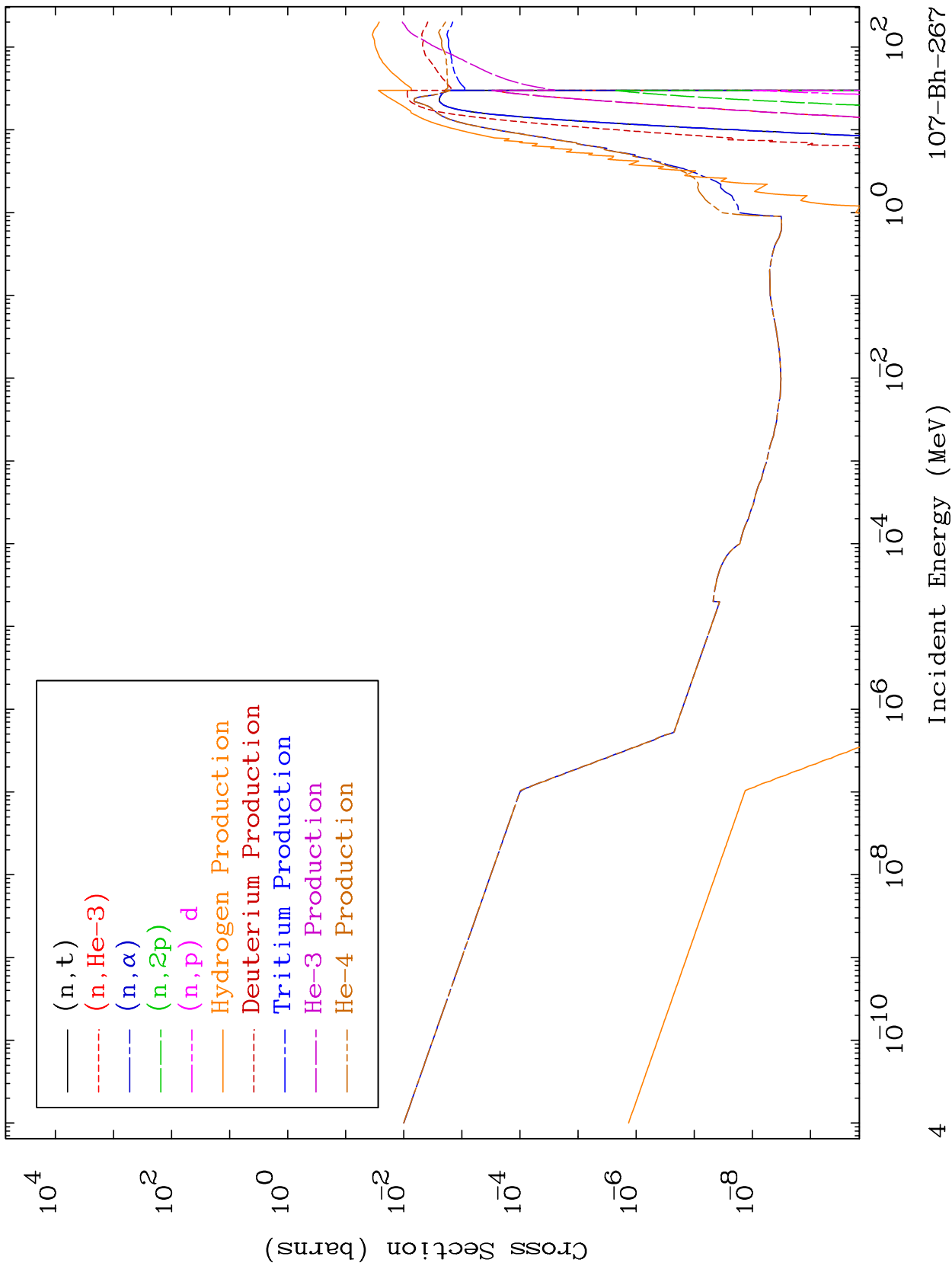




MAT 767

Neutron Absorption
293 Kelvin Cross Sections

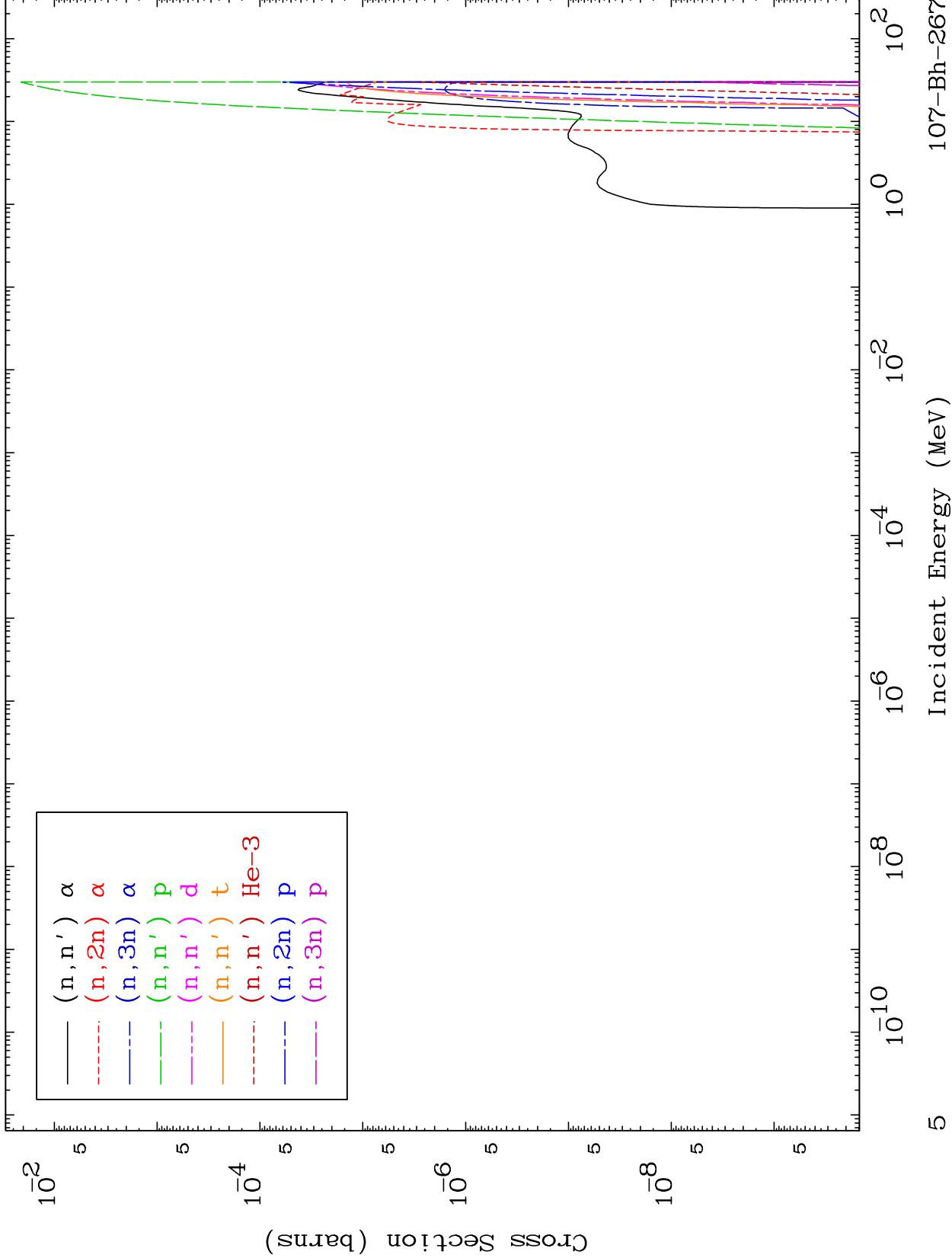
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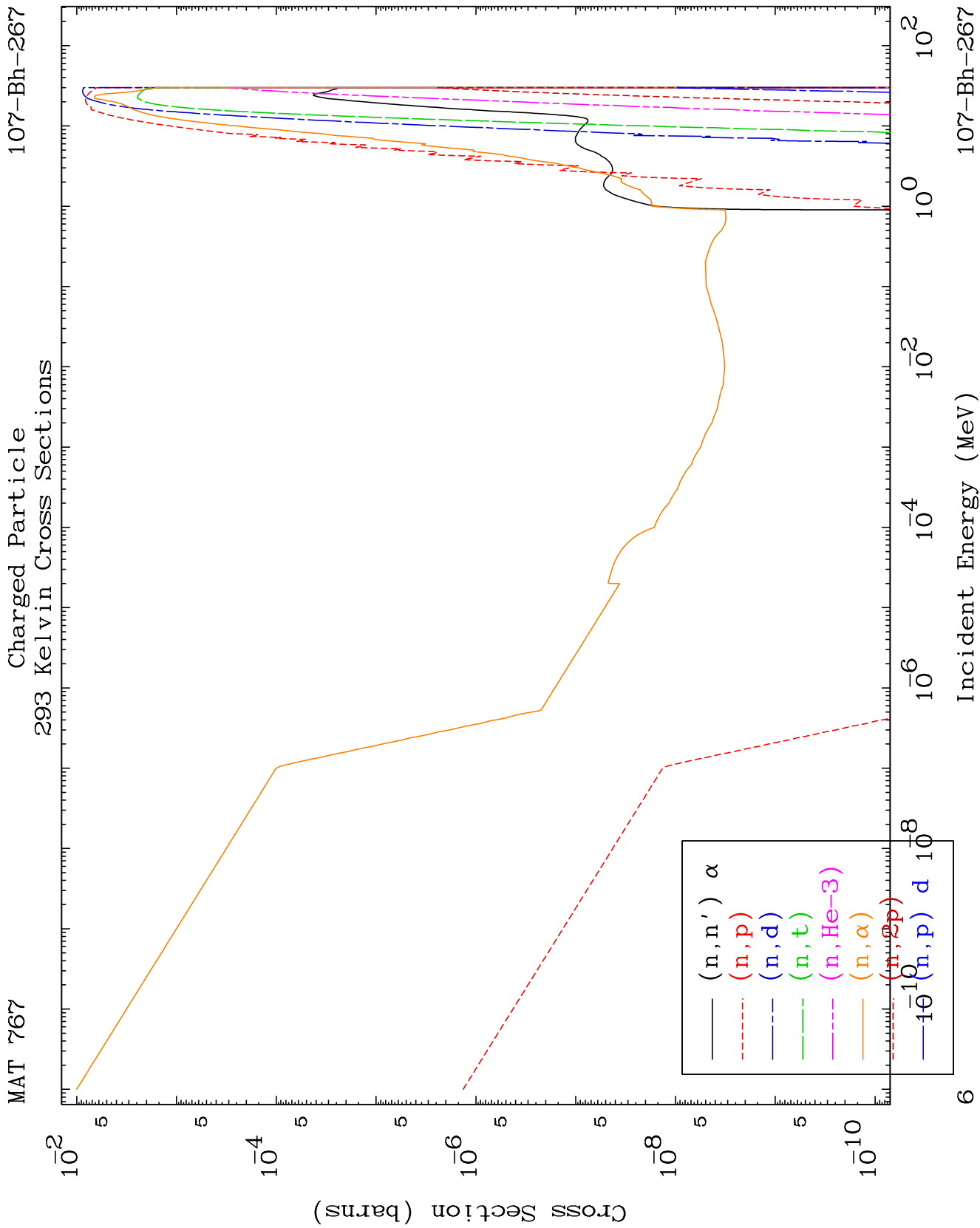


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Charged Particle
293 Kelvin Cross Sections

107-Bh-267

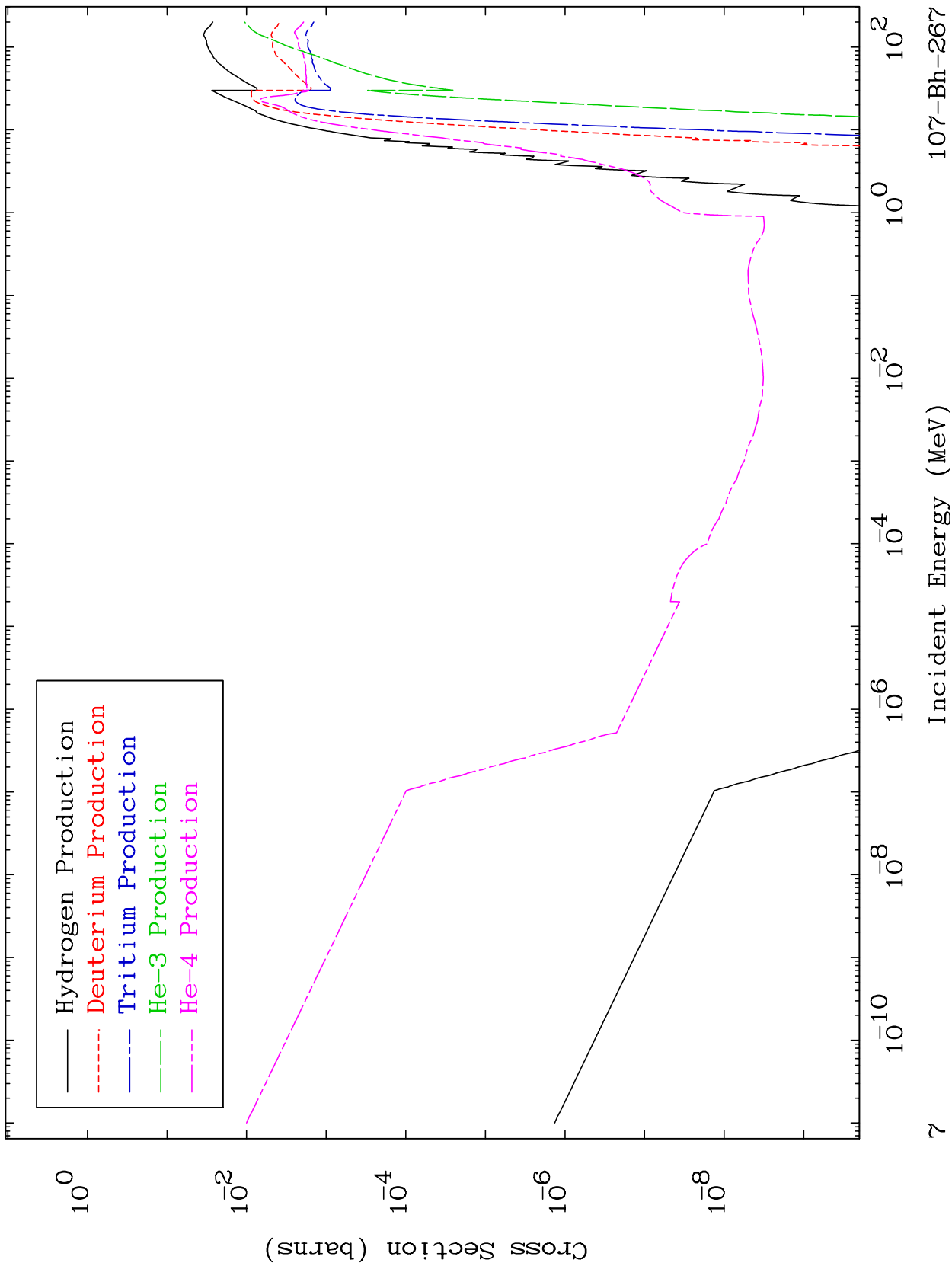




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Particle Production
293 Kelvin Cross Sections

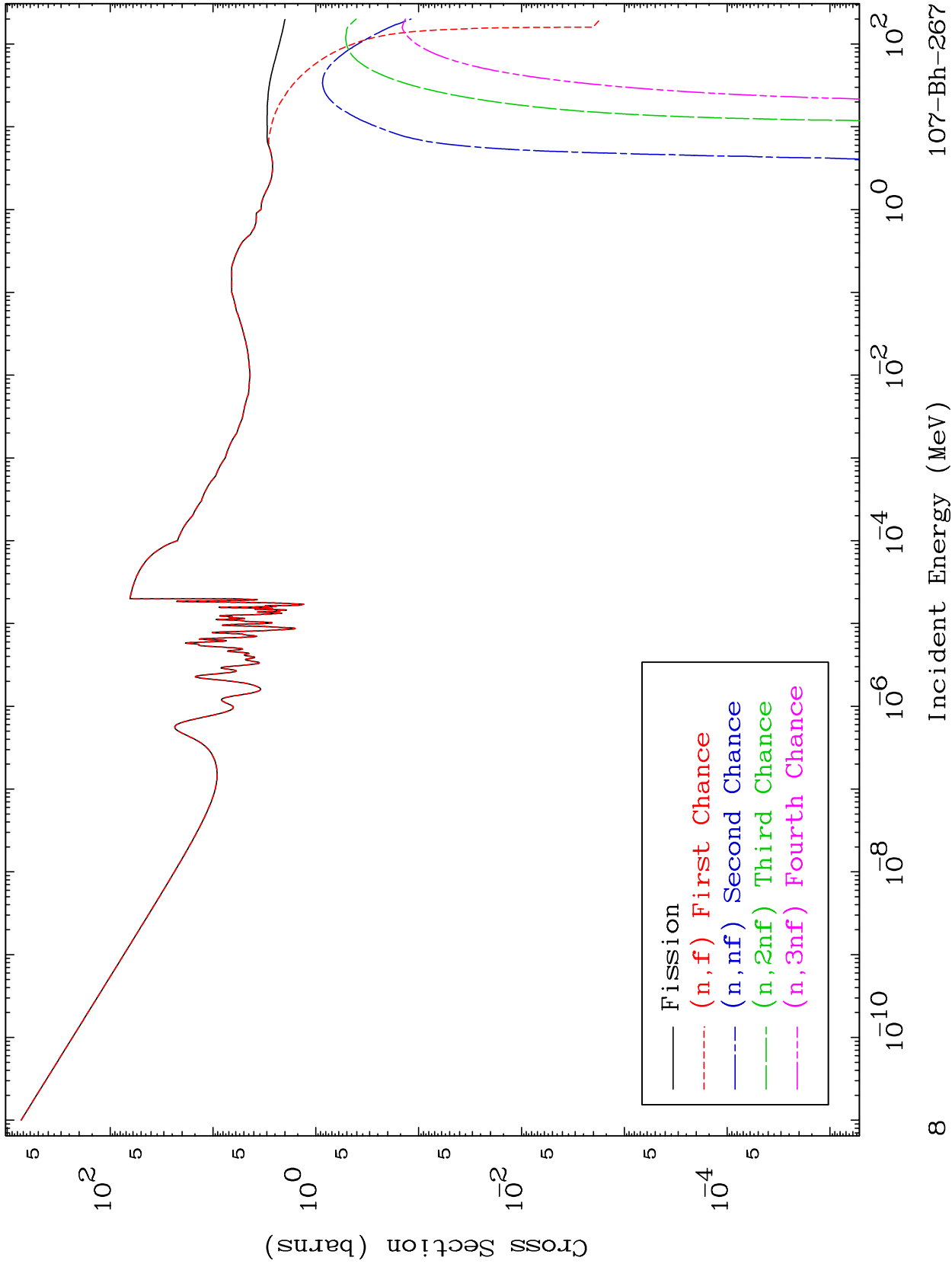
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MAT 767

Fission
293 Kelvin Cross Sections

107-Bh-267

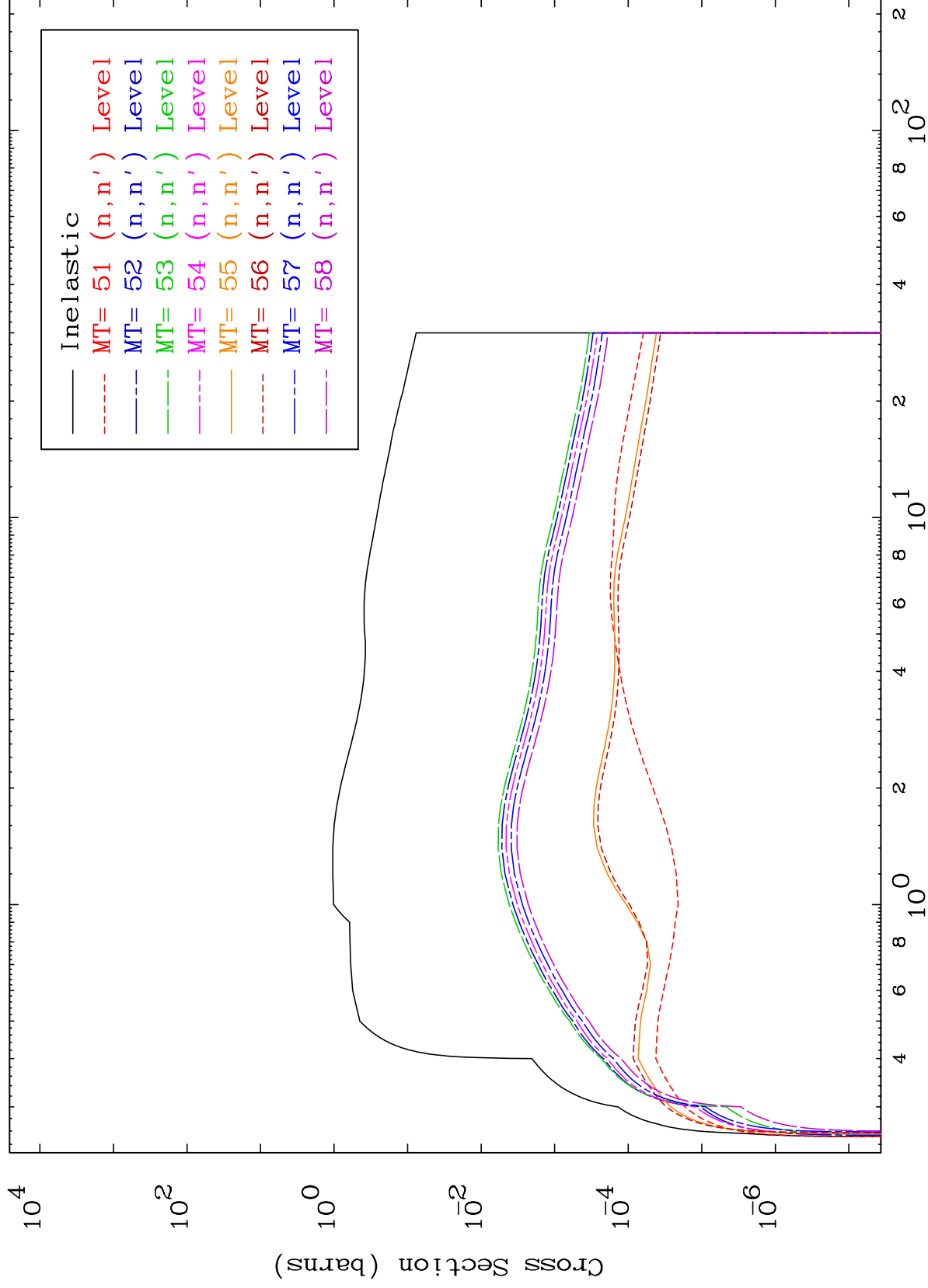


MAT 767

(n,n') Levels

293 Kelvin Cross Sections

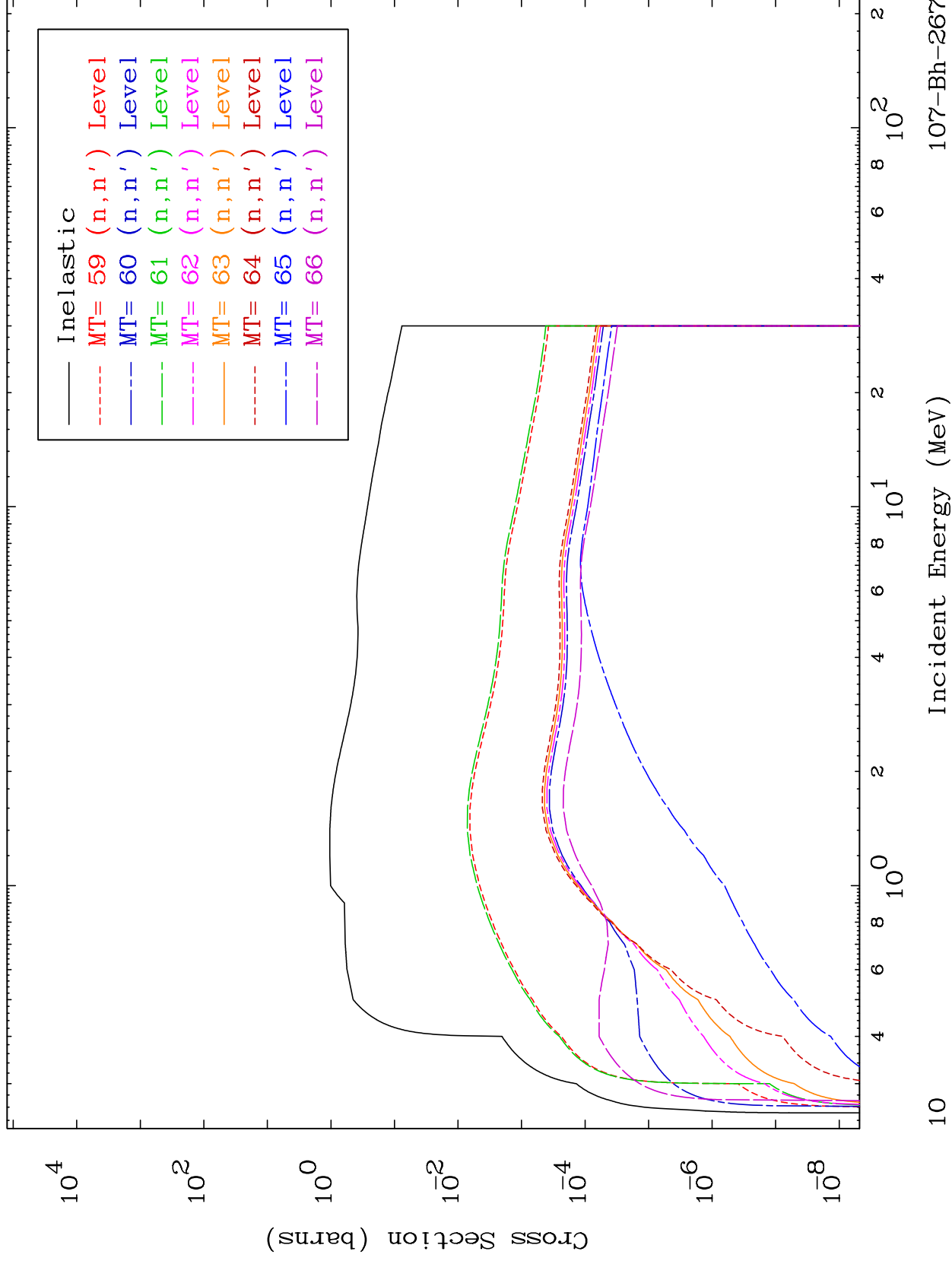
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(n,n') Levels
293 Kelvin Cross Sections

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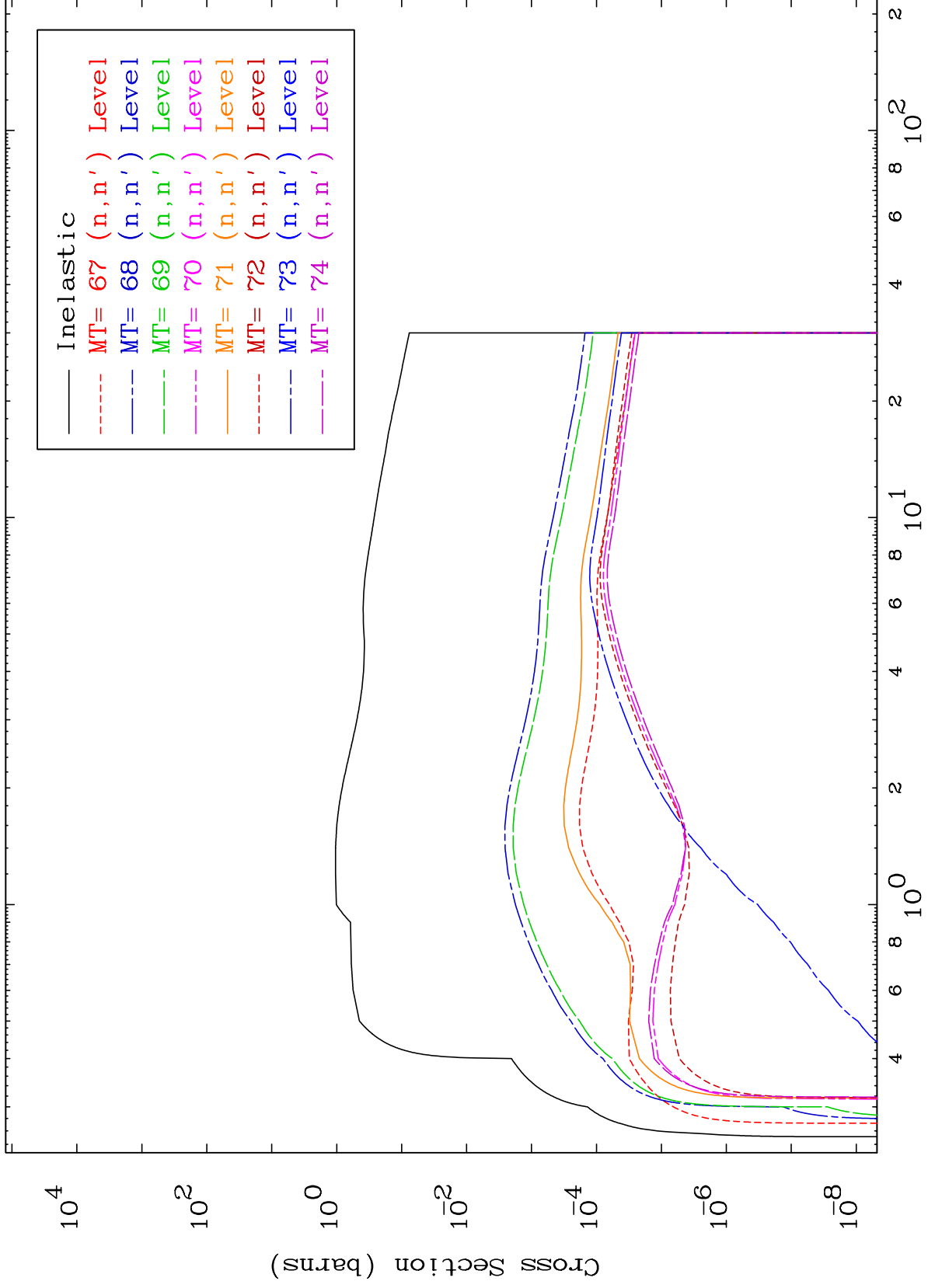
Incident Energy (MeV)

10

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(n,n') Levels
293 Kelvin Cross Sections

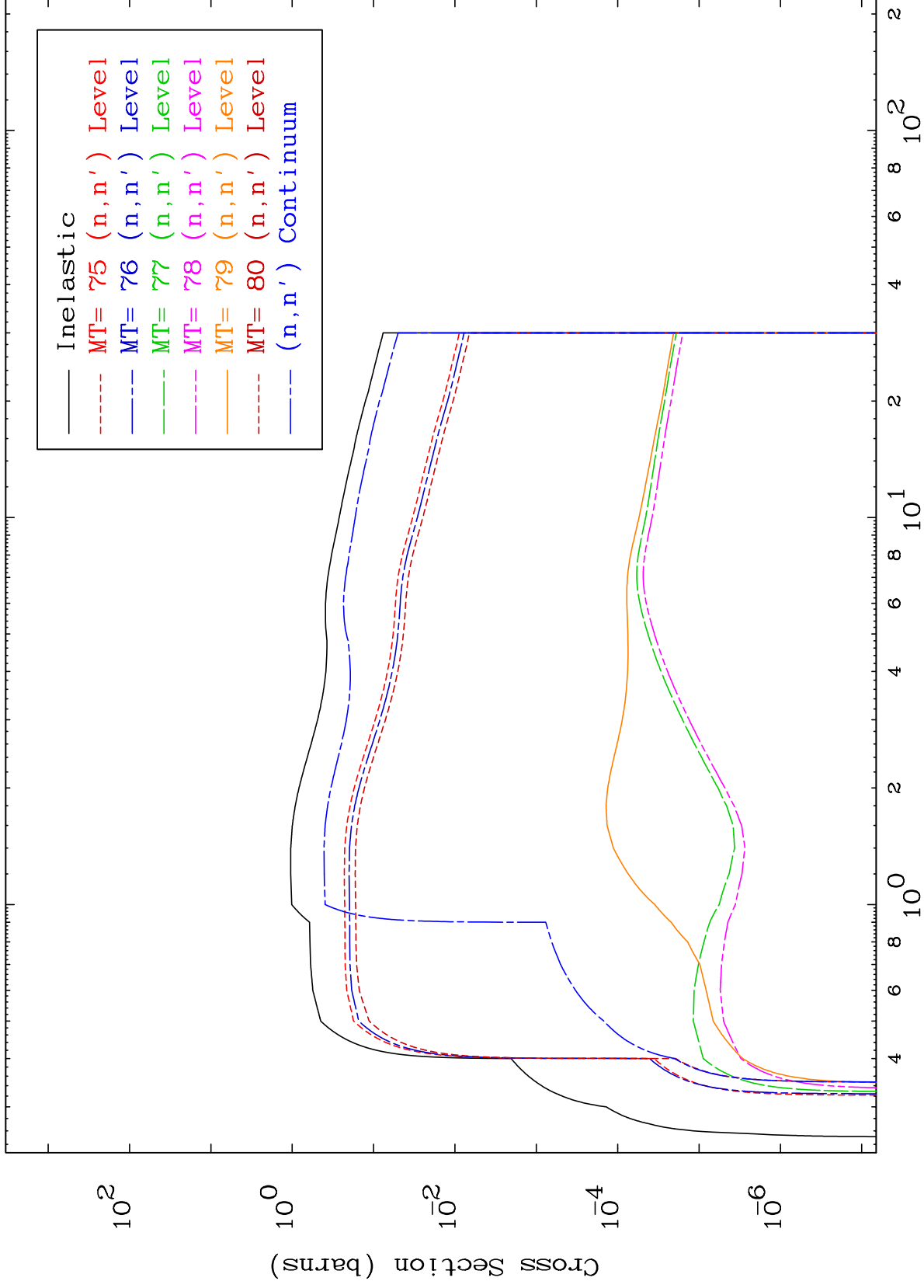
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(n,n') Levels
293 Kelvin Cross Sections

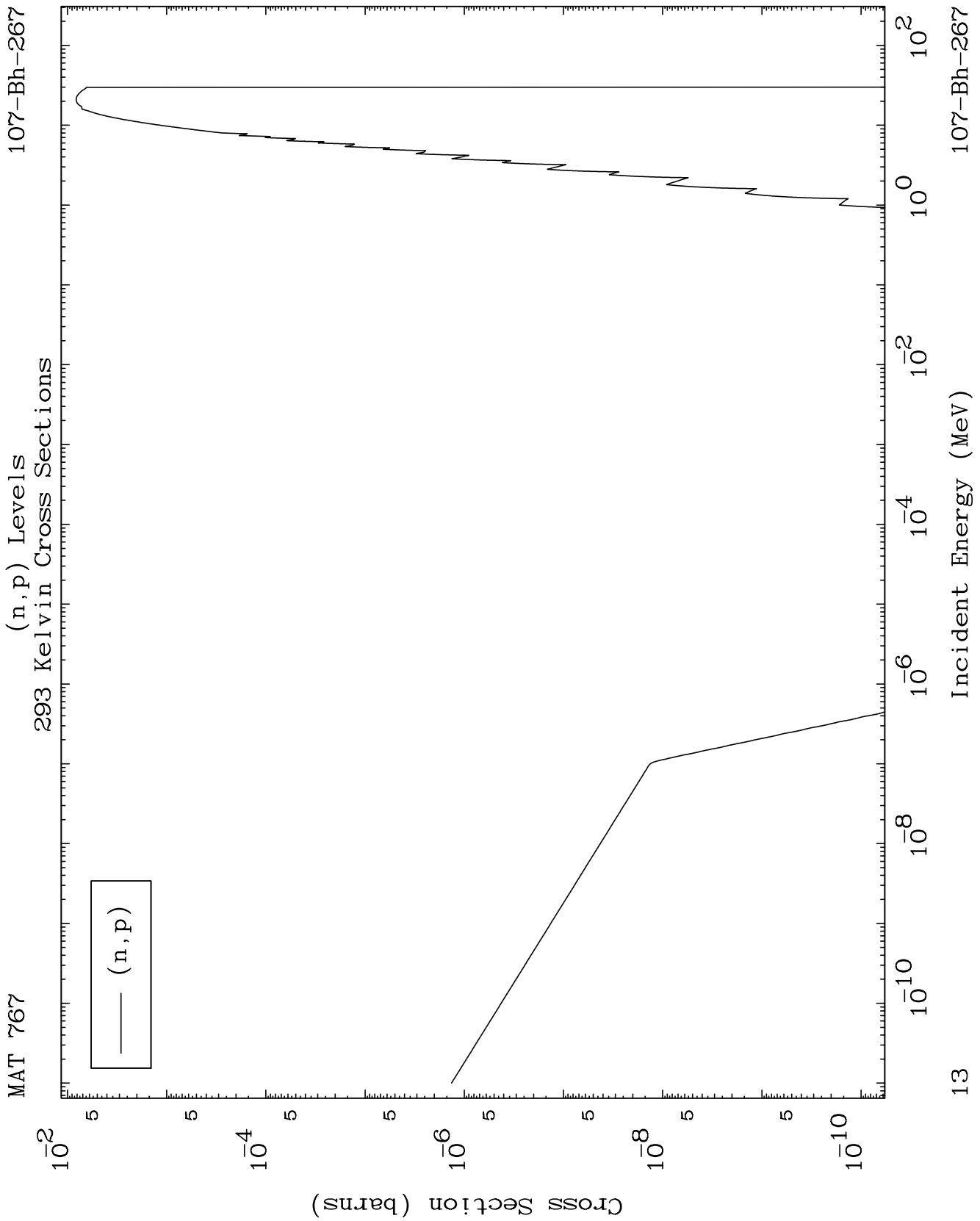
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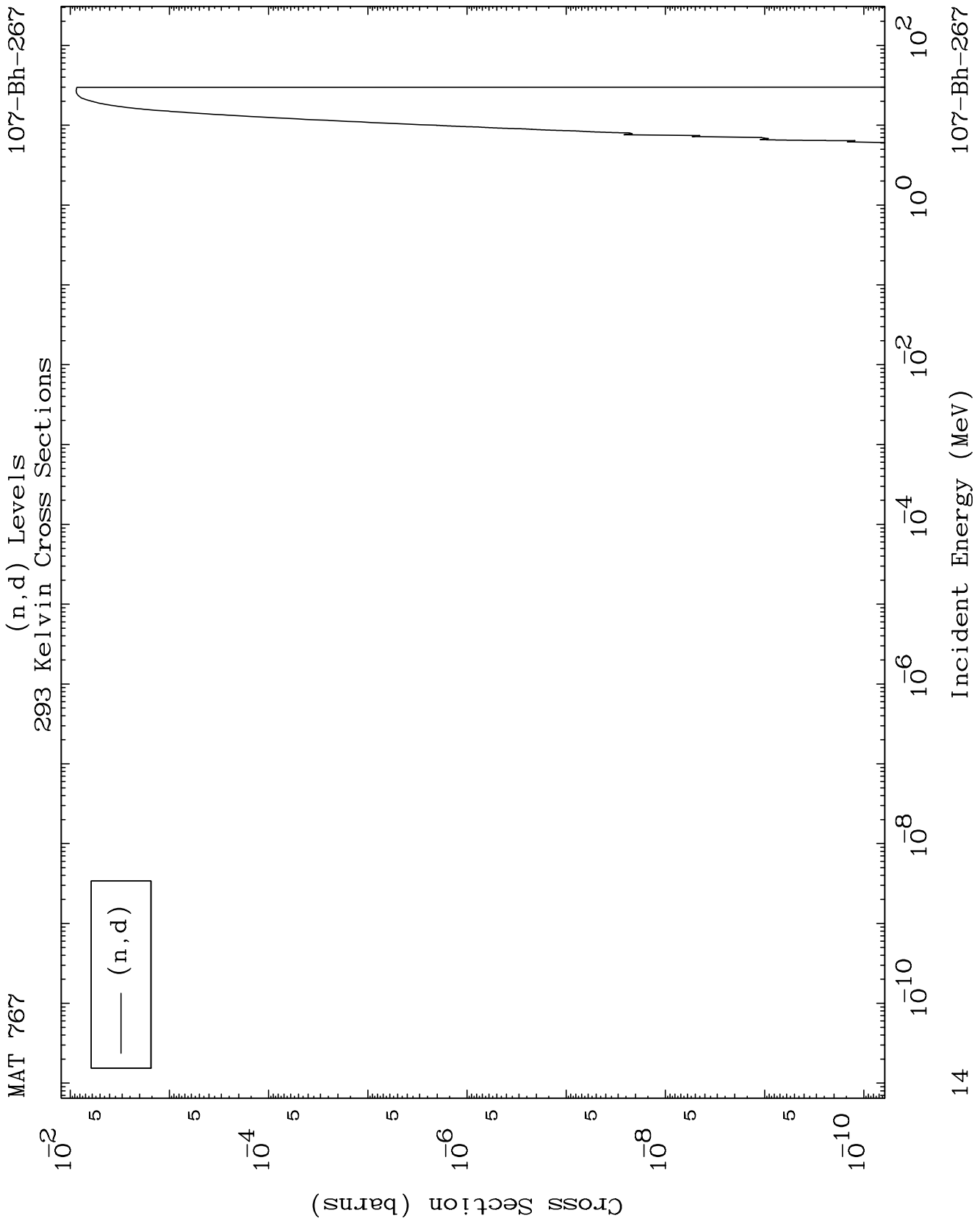


12

Incident Energy (MeV)

107-Bh-267

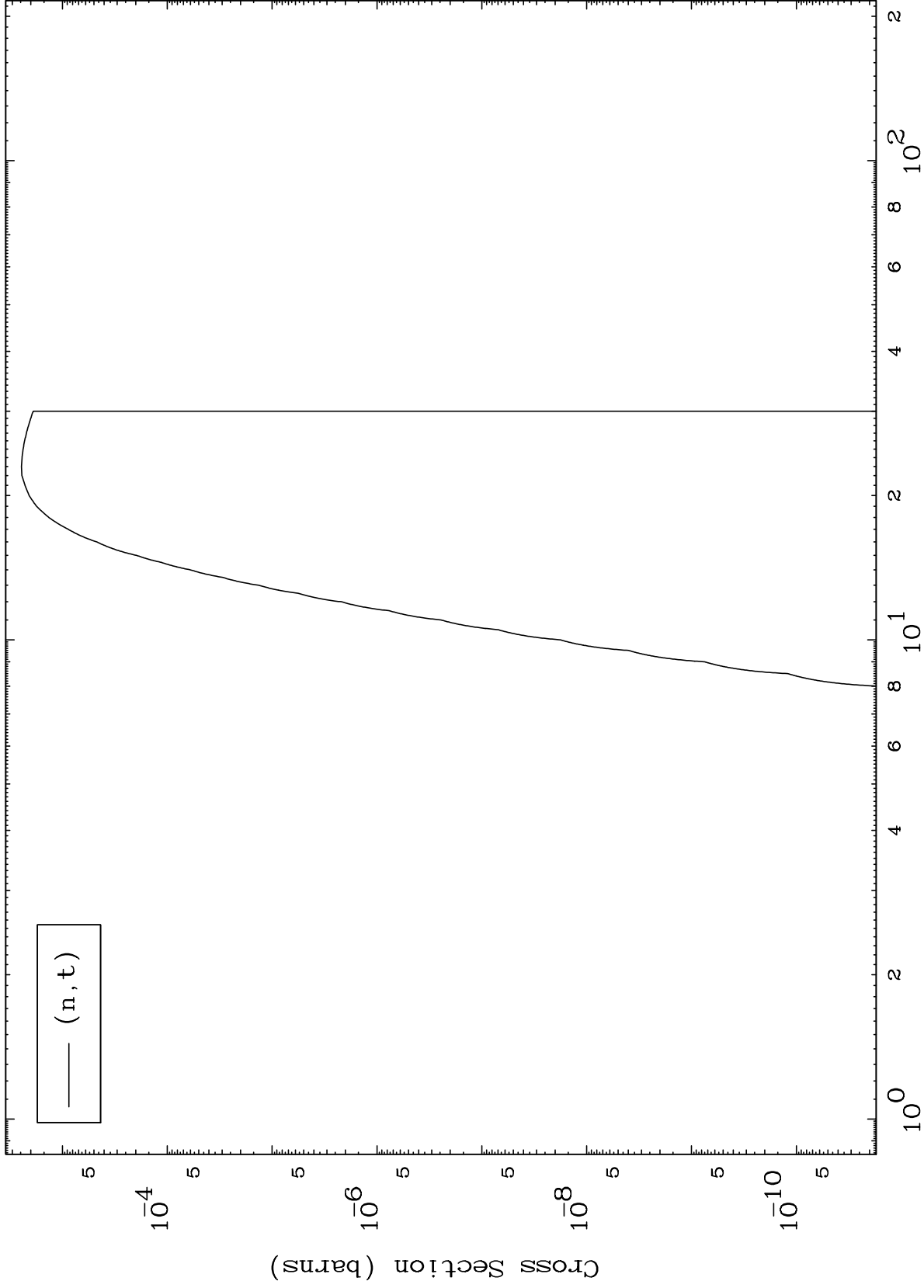




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(n,t) Levels
293 Kelvin Cross Sections

107-Bh-267



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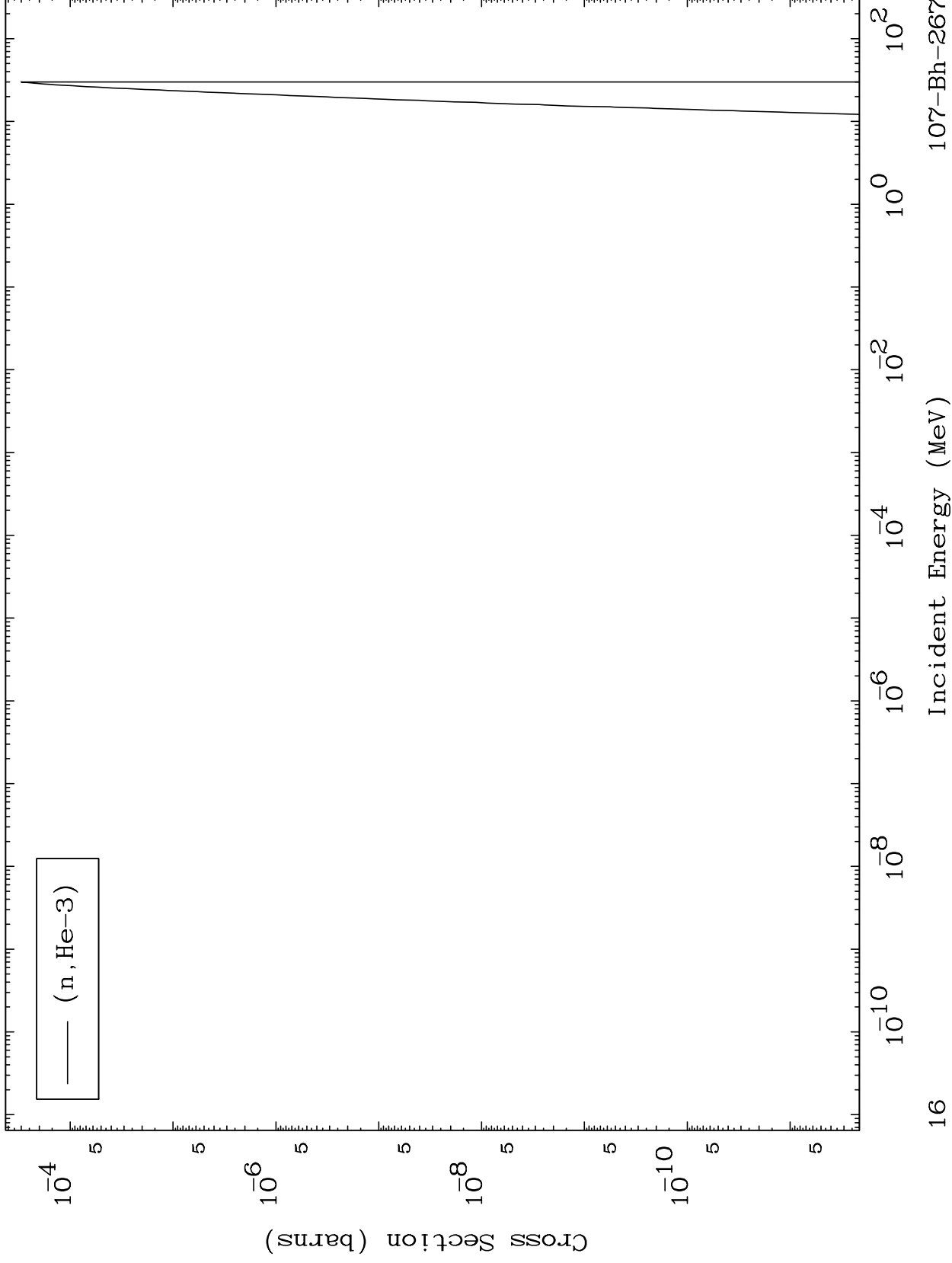
Incident Energy (MeV)

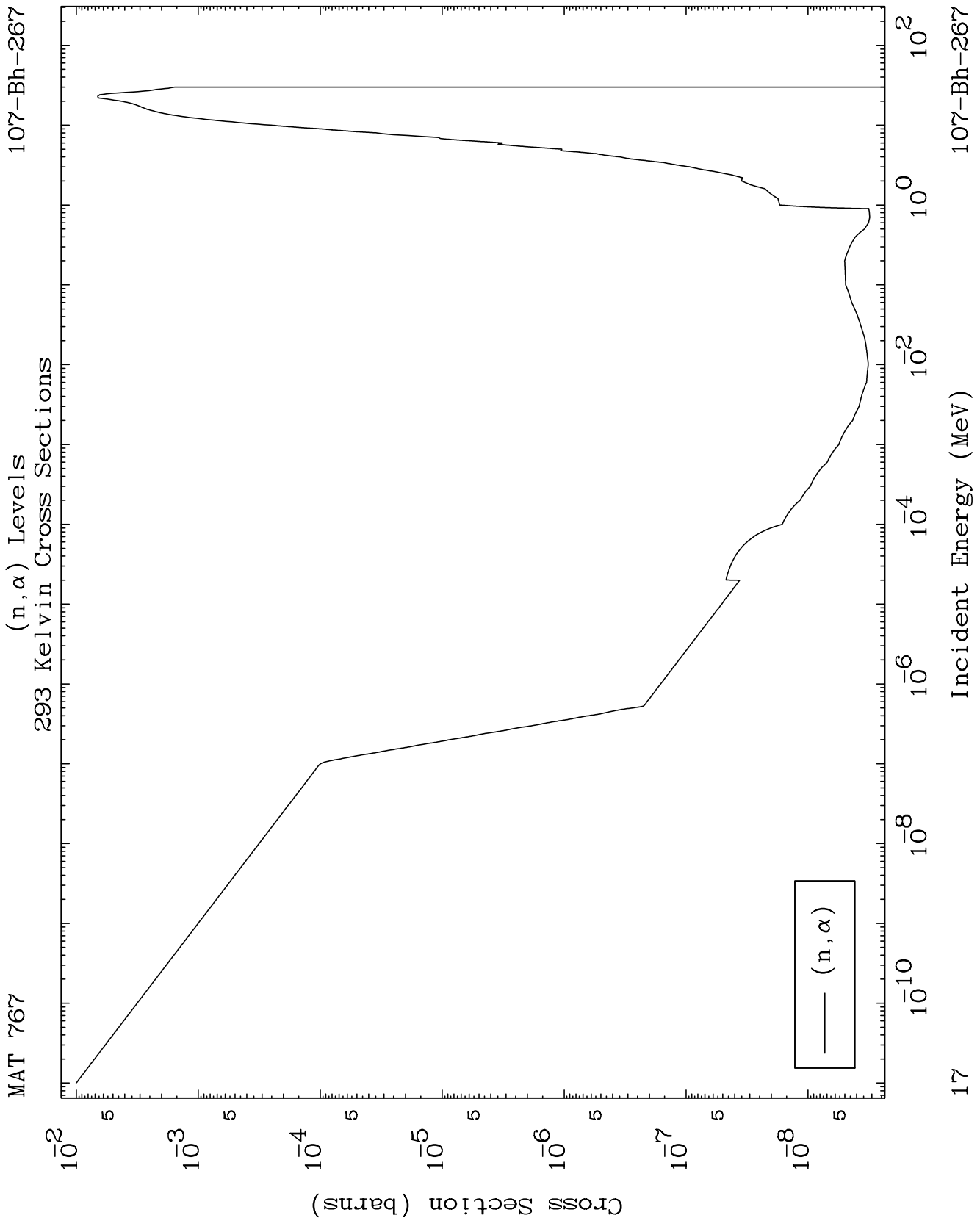
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(n,He3) Levels
293 Kelvin Cross Sections

107-Bh-267

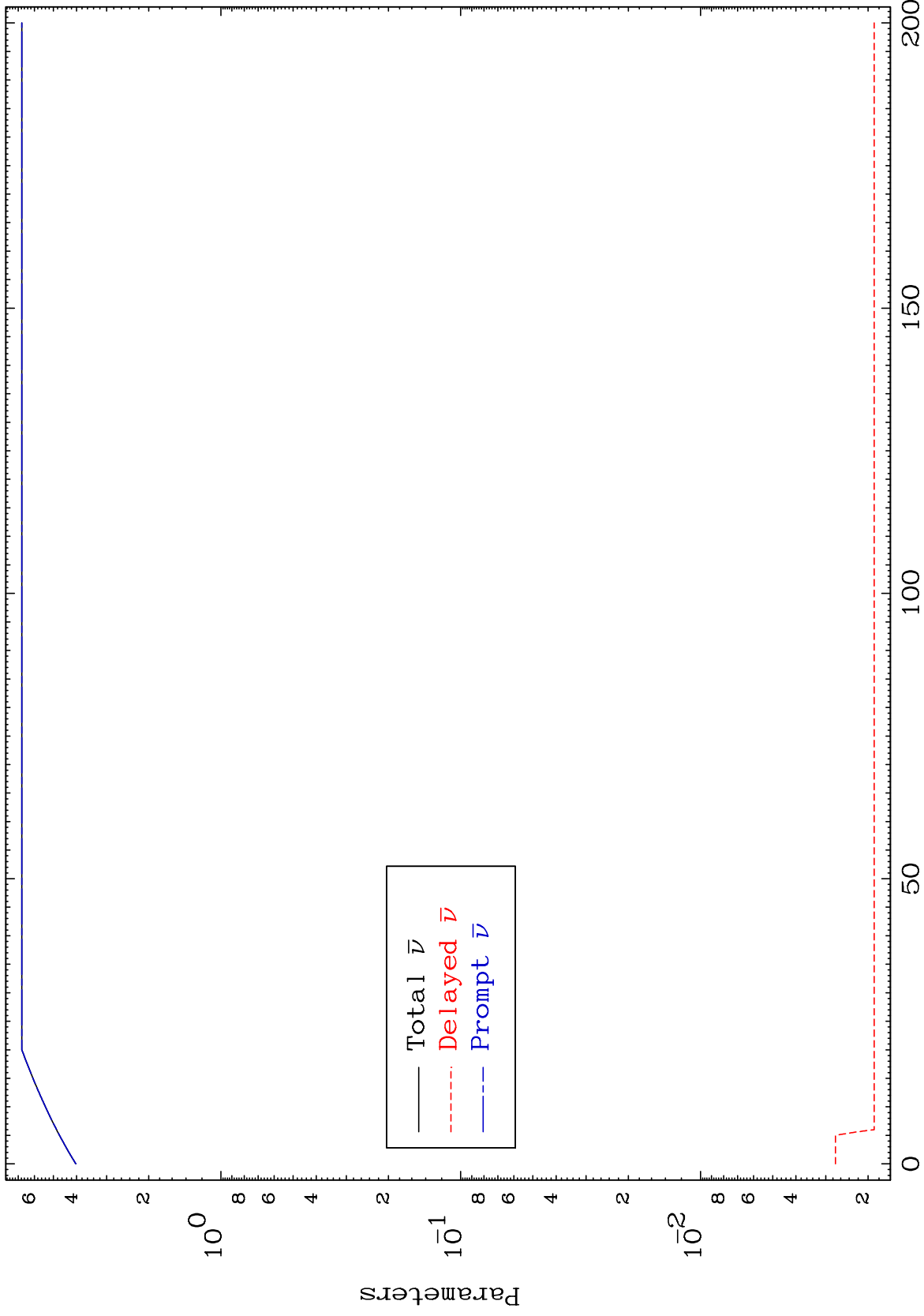




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Energy Release
Parameters

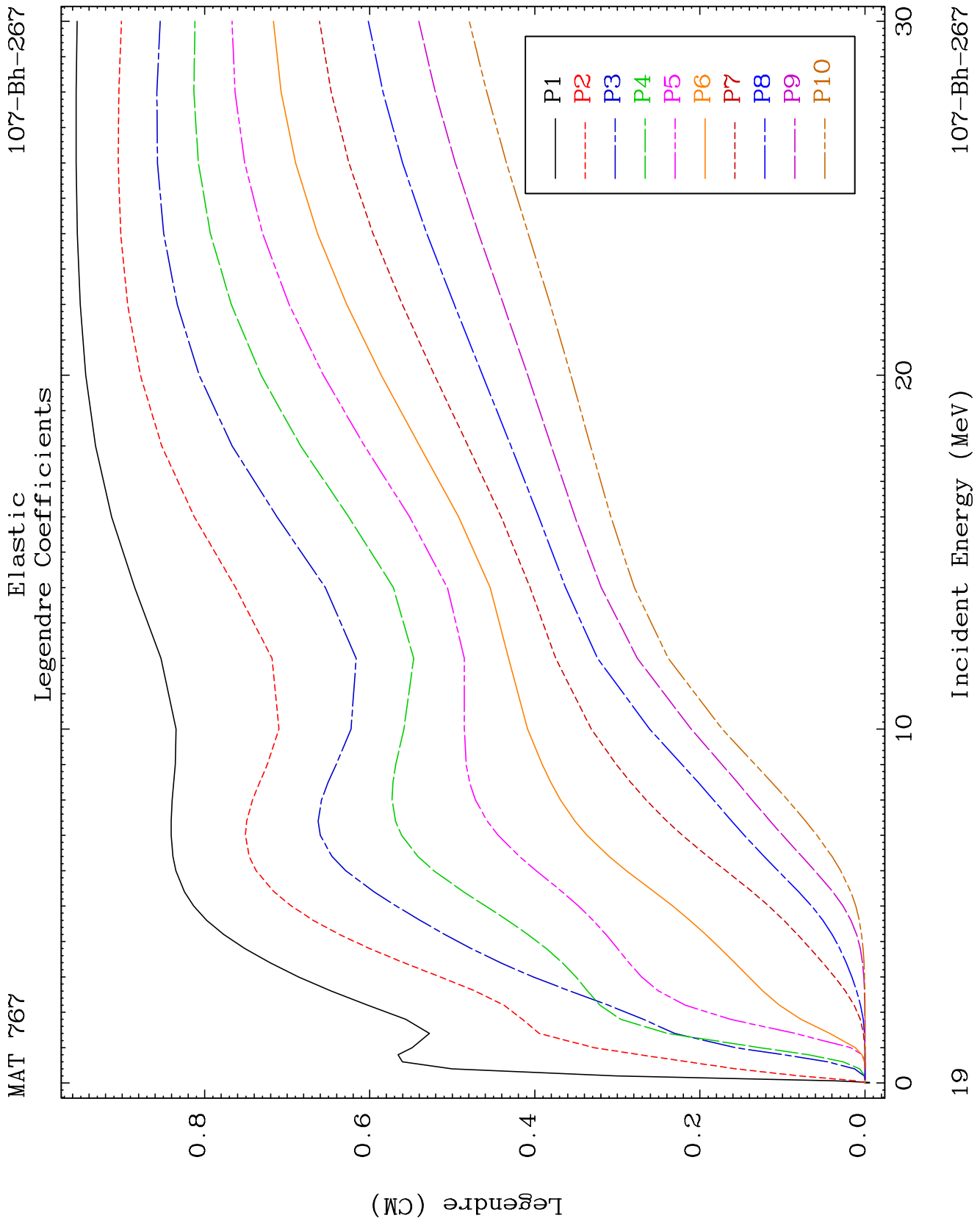
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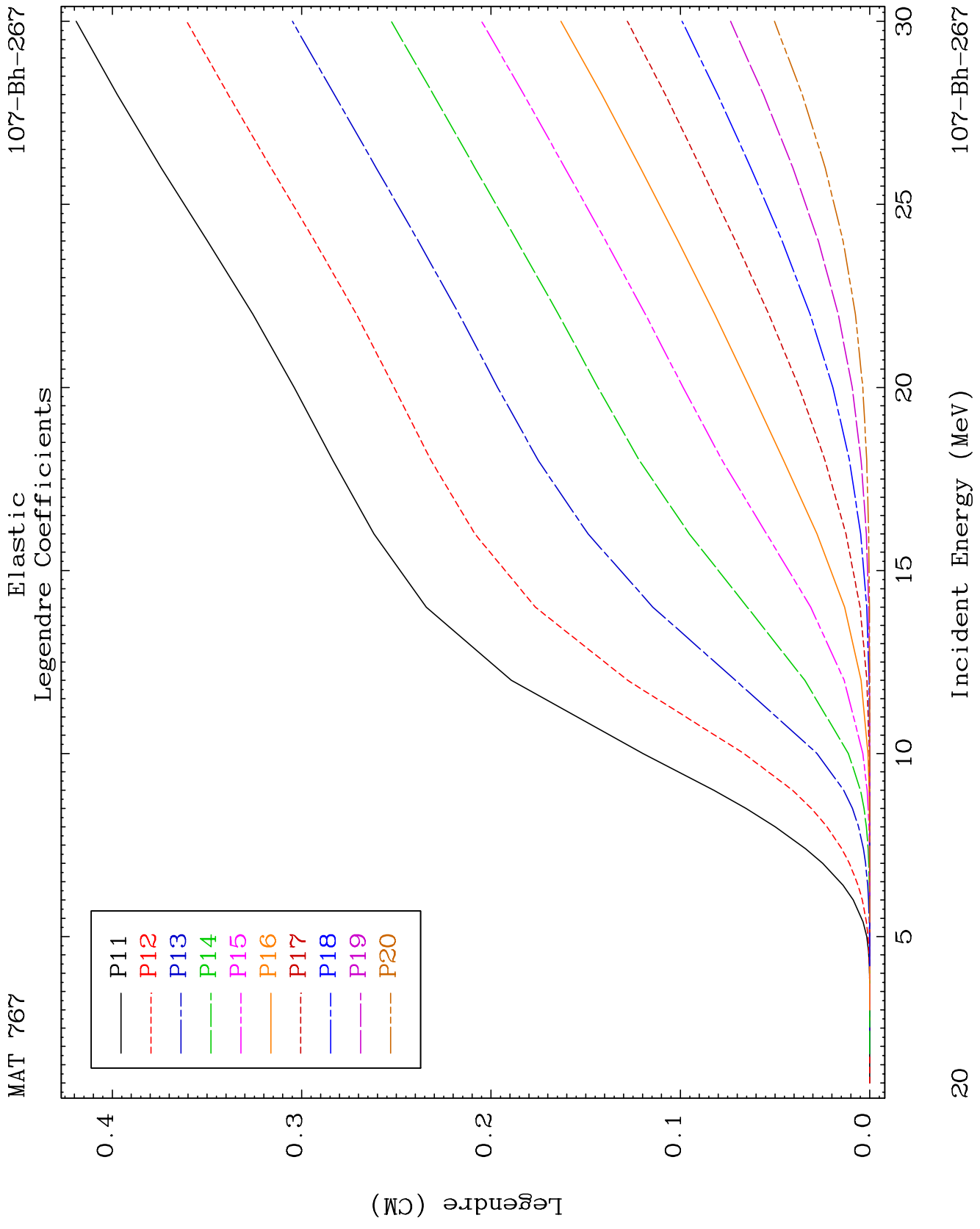


18

Incident Energy (MeV)

107-Bh-267

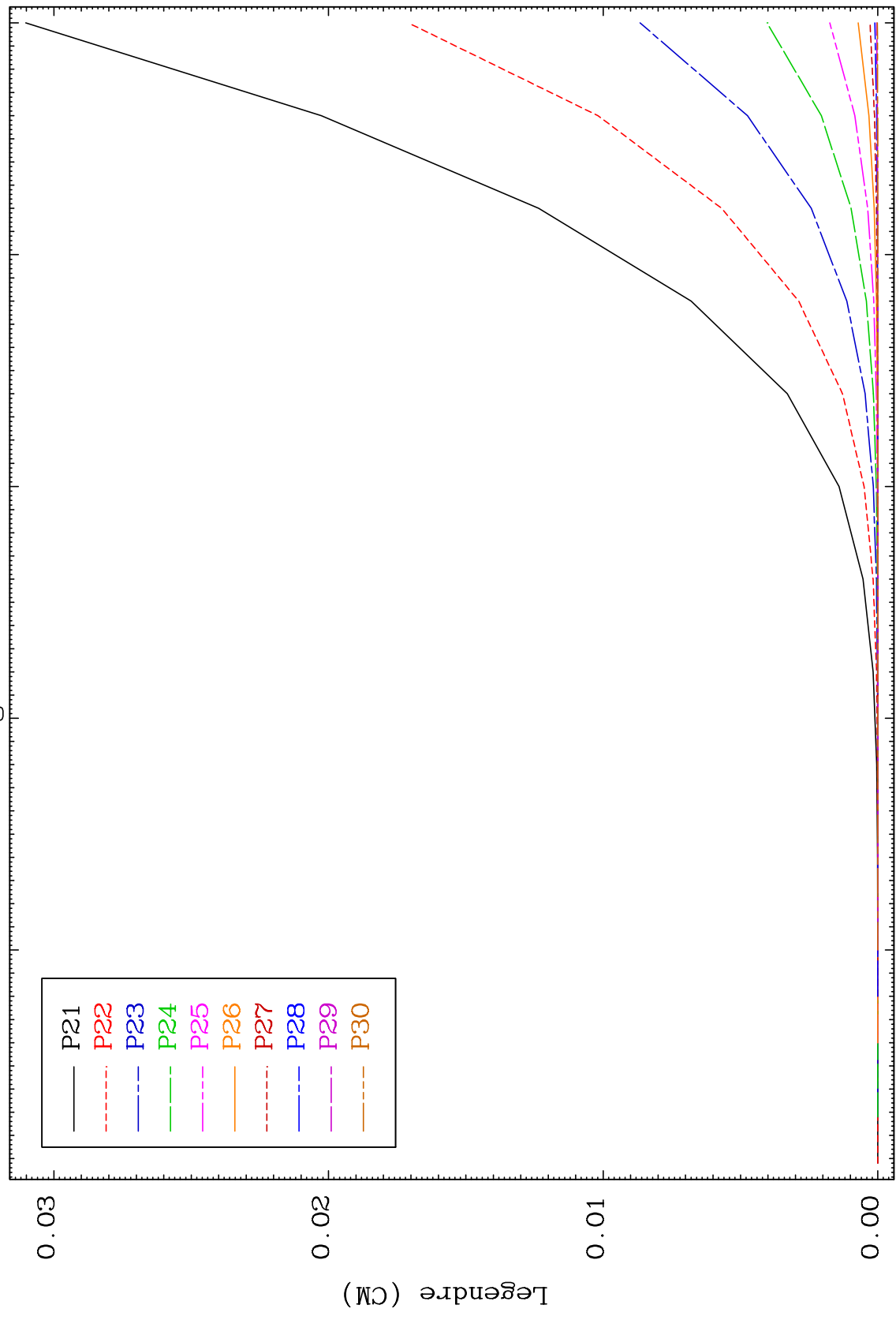




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Elastic Legendre Coefficients

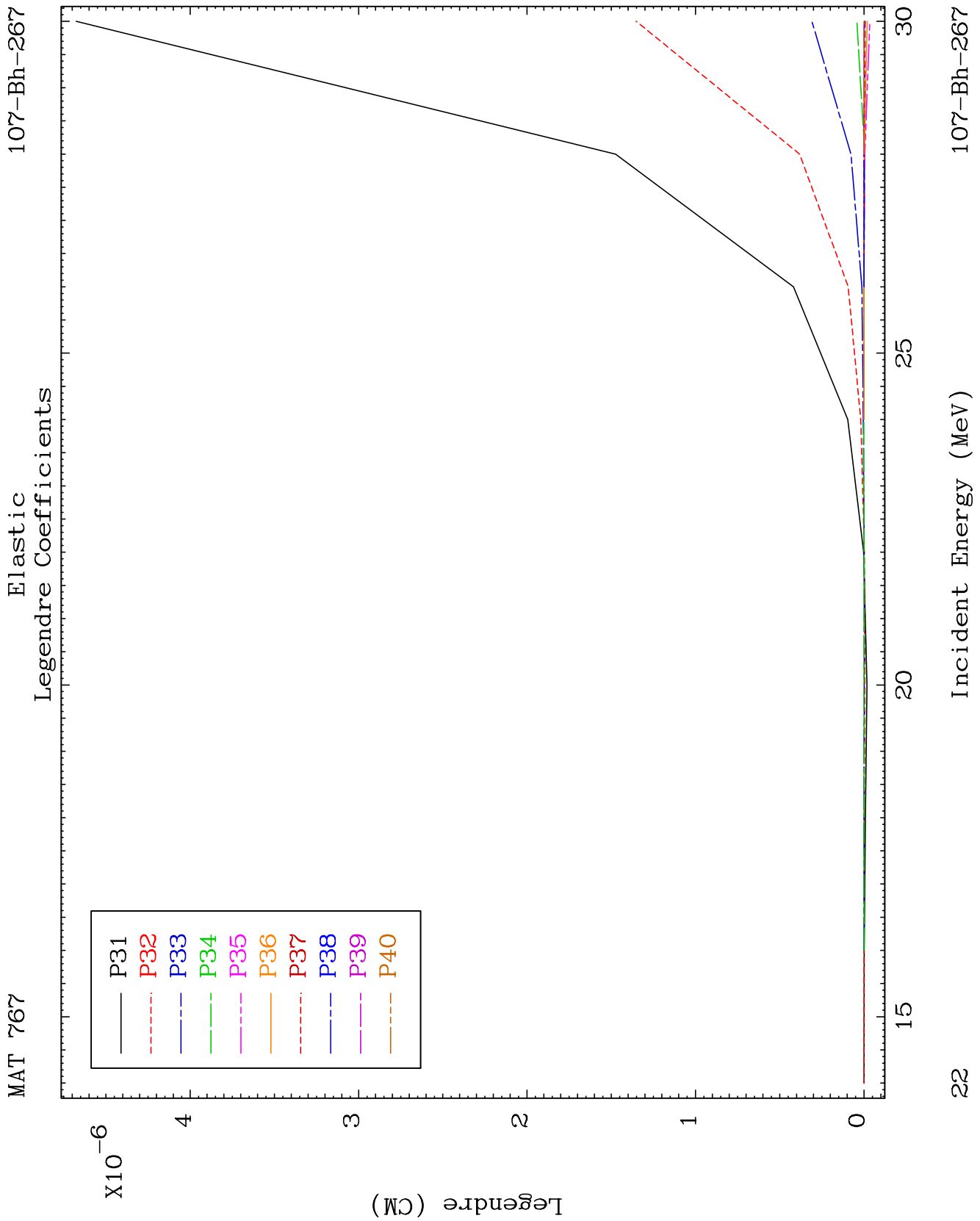
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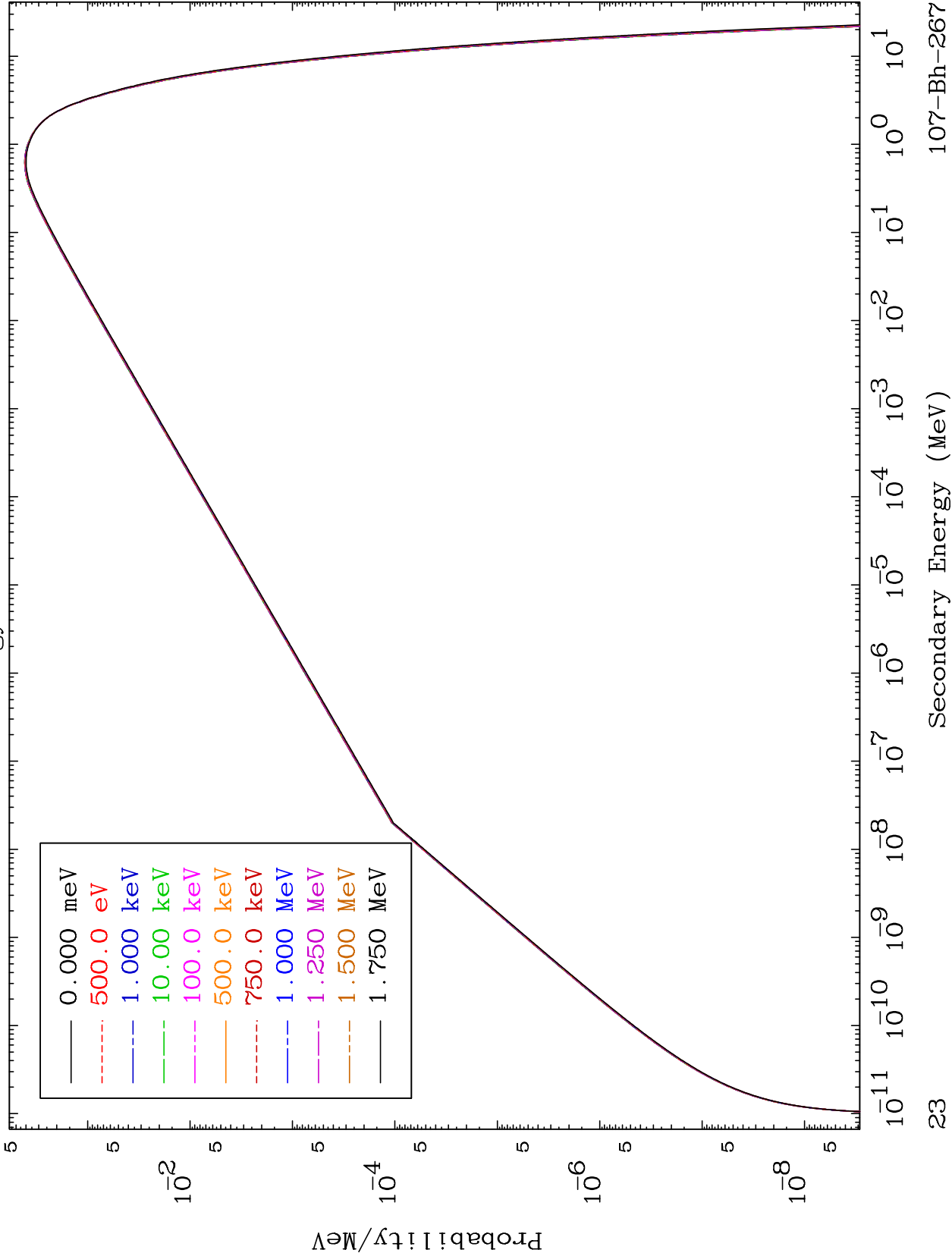
21

Incident Energy (MeV)

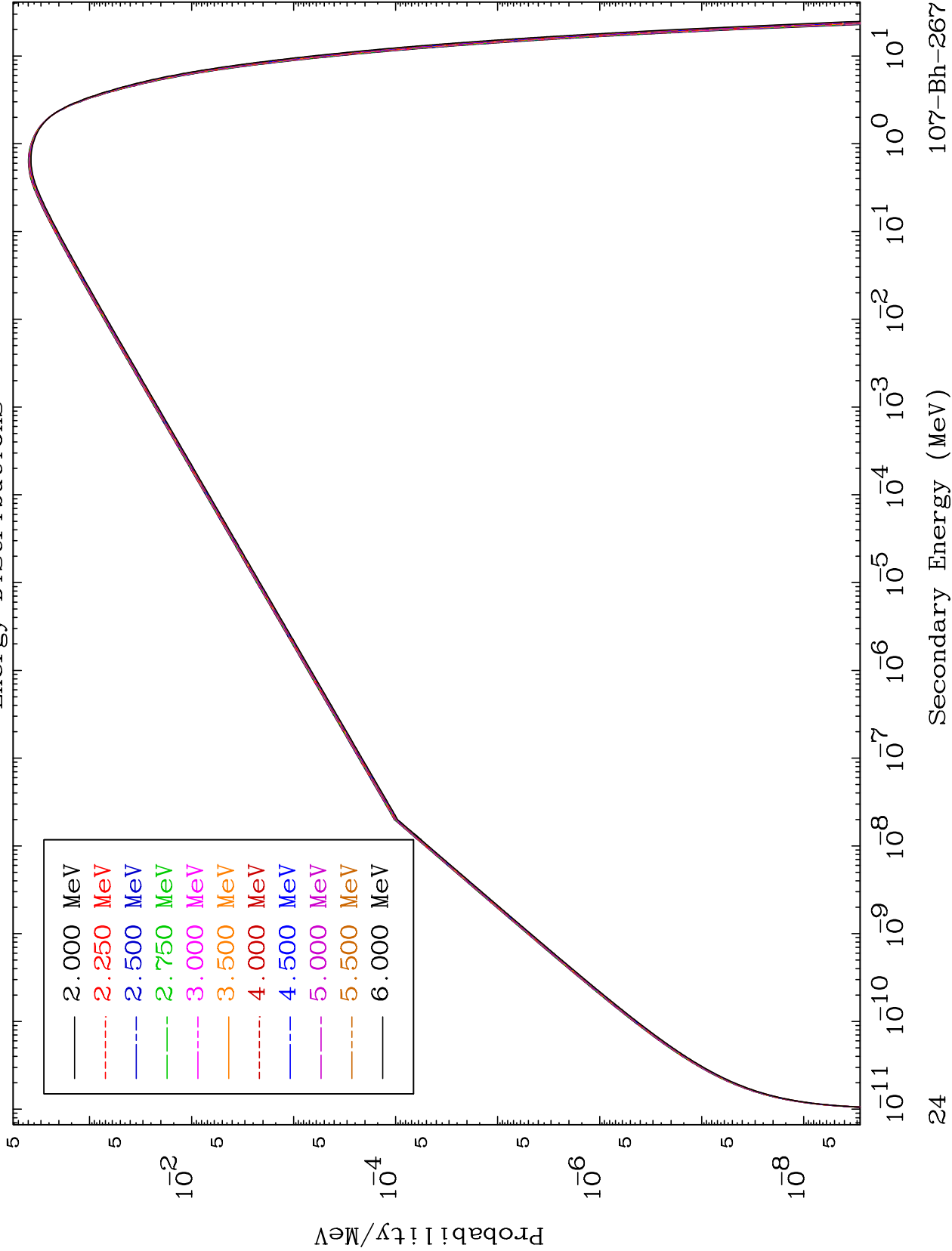
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Fission Energy Distributions



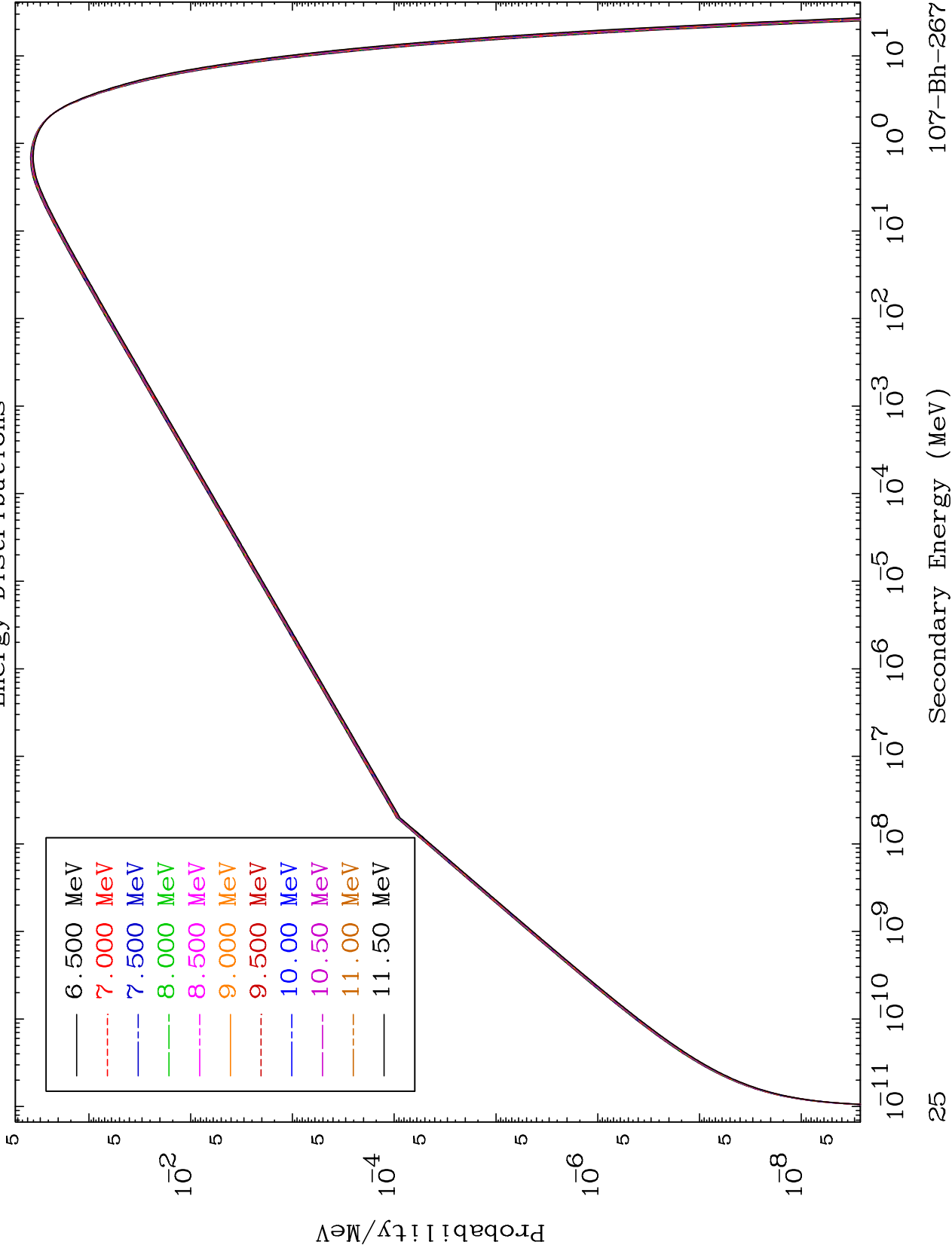
Fission Energy Distributions



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Fission Energy Distributions

107-Bh-267

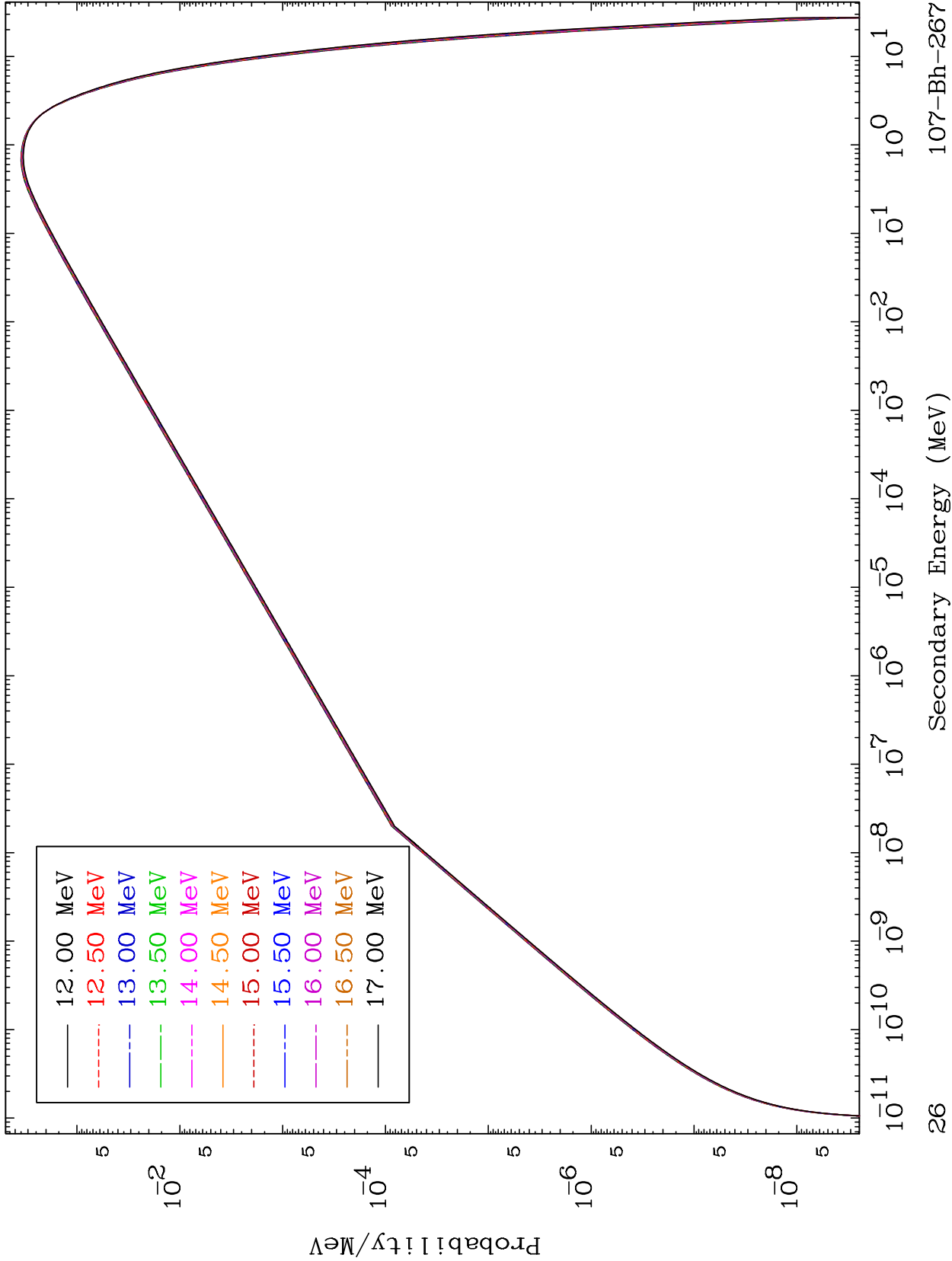


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Secondary Energy (MeV)

25

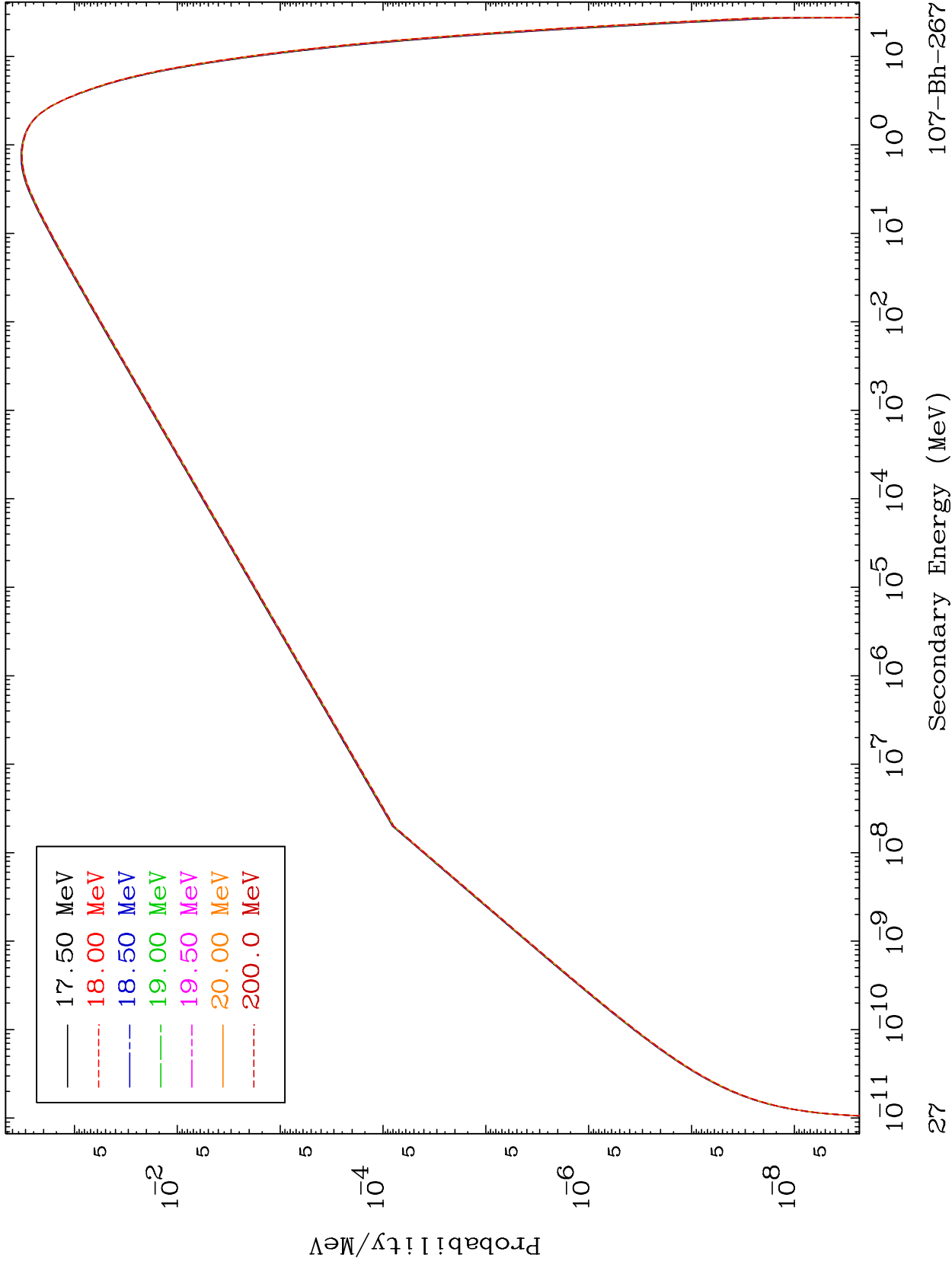
Fission Energy Distributions



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Fission Energy Distributions

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Secondary Energy (MeV)

27