

Program EVALPLOT
(Version 2021-1)

by

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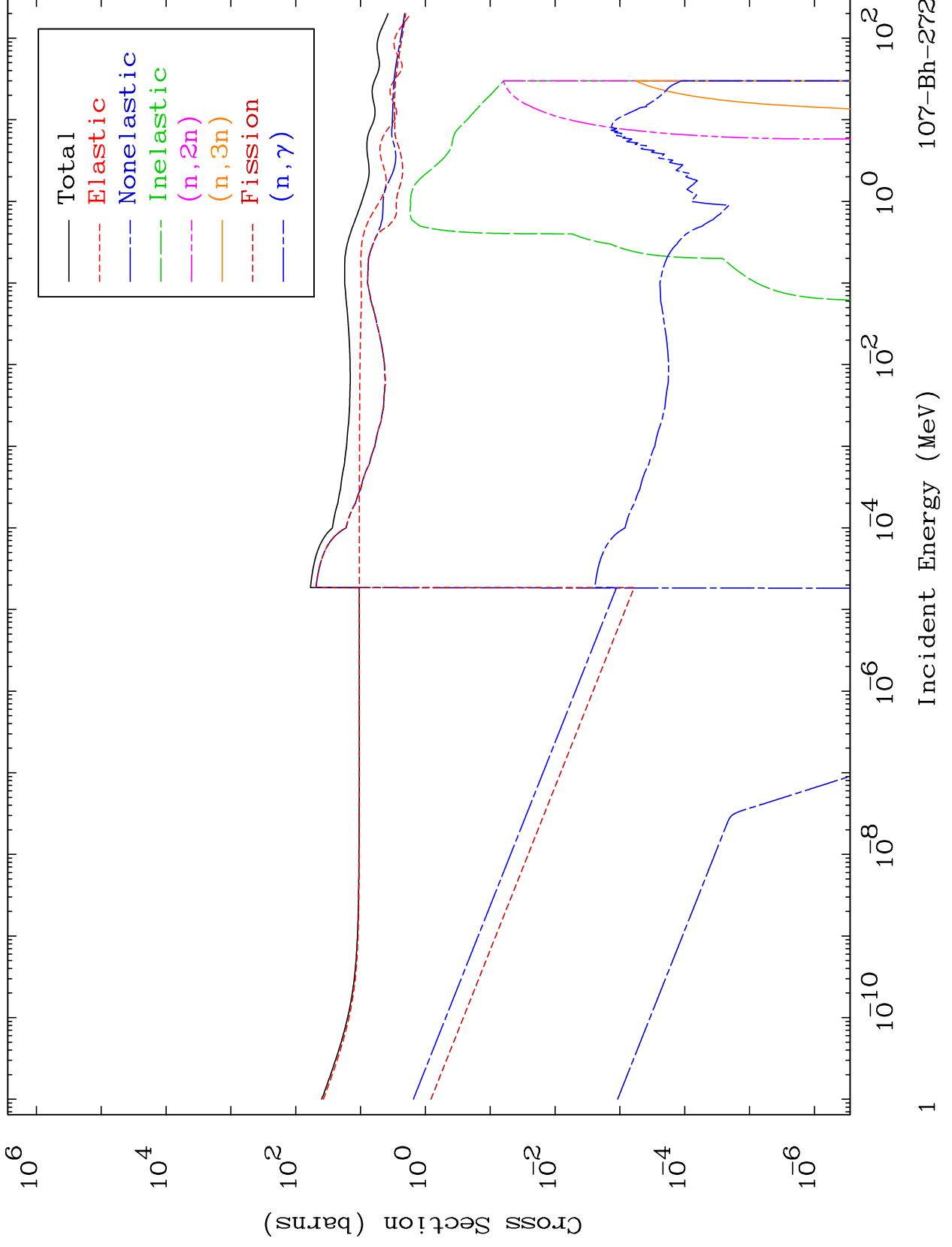
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Press Mouse Button to Start

MAT 772

Neutron Major
293 Kelvin Cross Sections

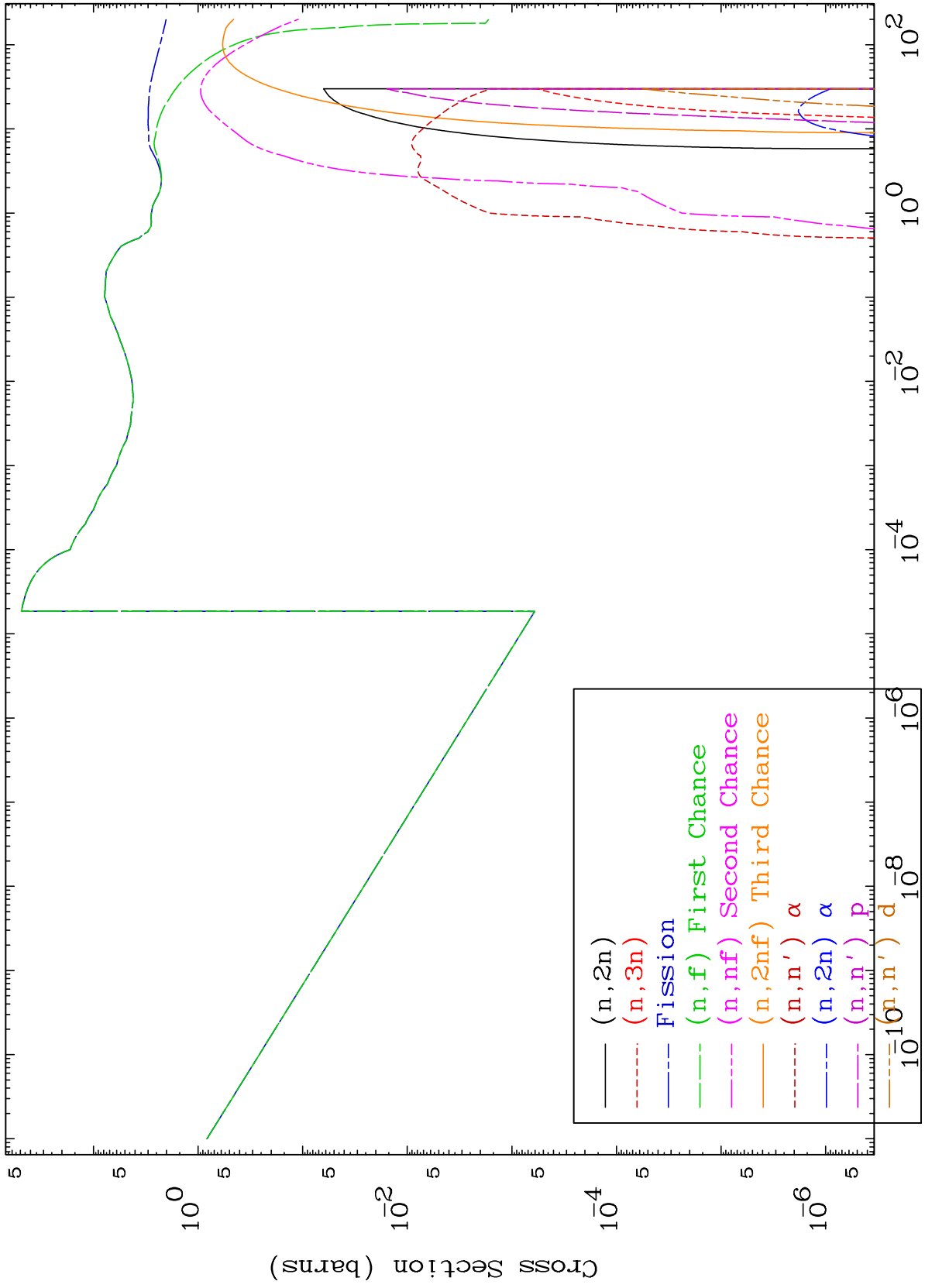
107-Bh-272



MAT 772

Neutron Absorption
293 Kelvin Cross Sections

107-Bh-272



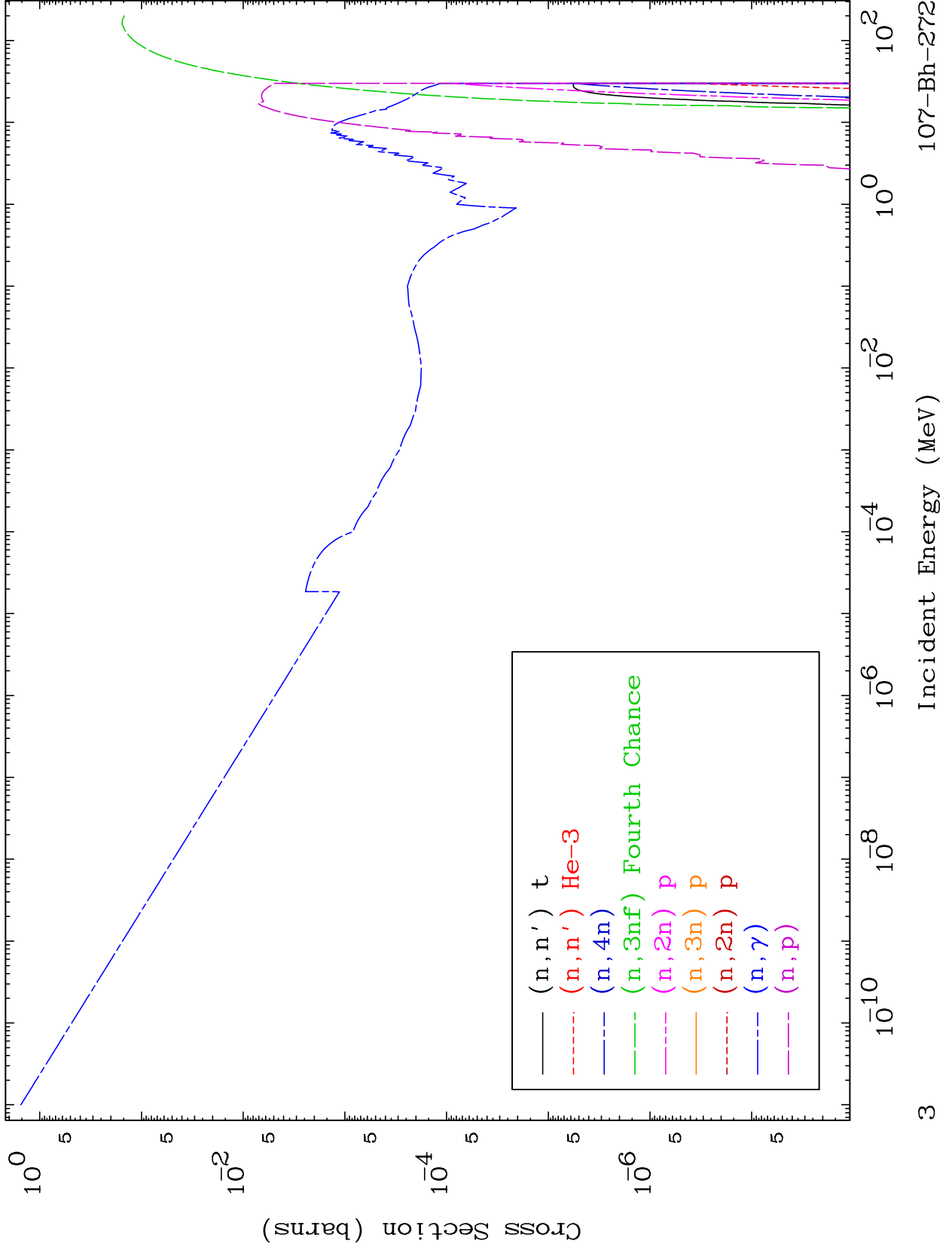
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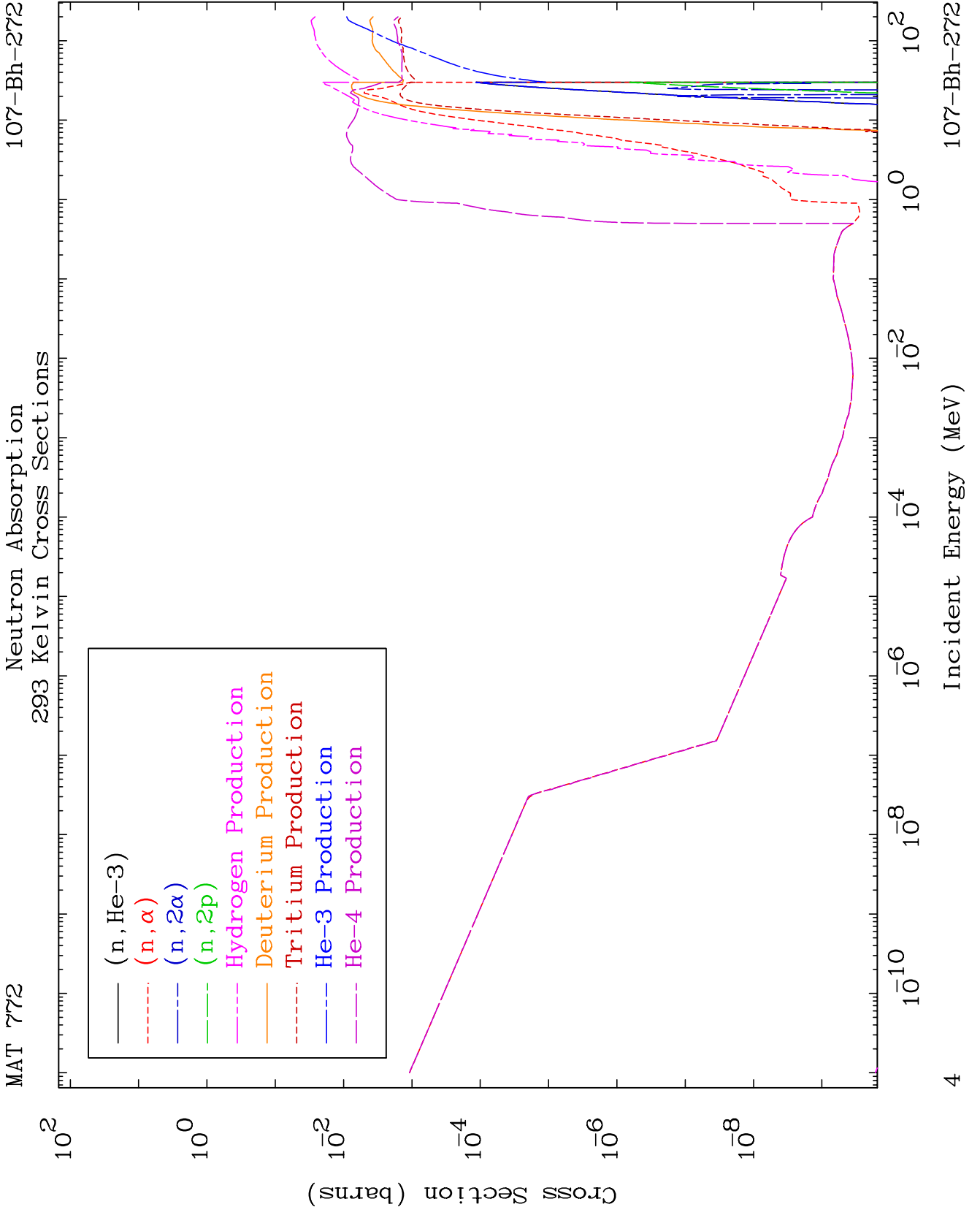
Incident Energy (MeV)

MAT 772

Neutron Absorption
293 Kelvin Cross Sections

107-Bh-272

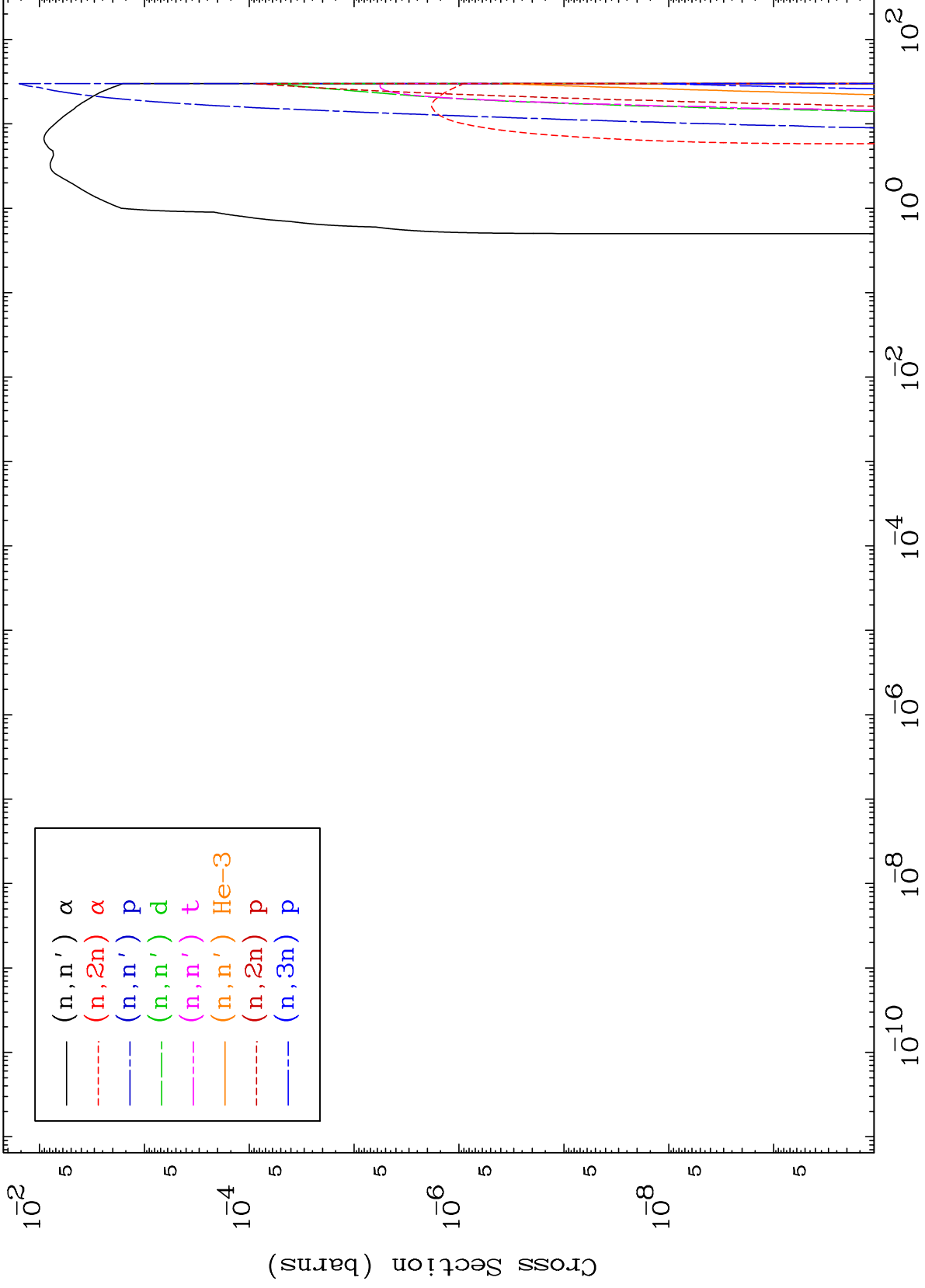


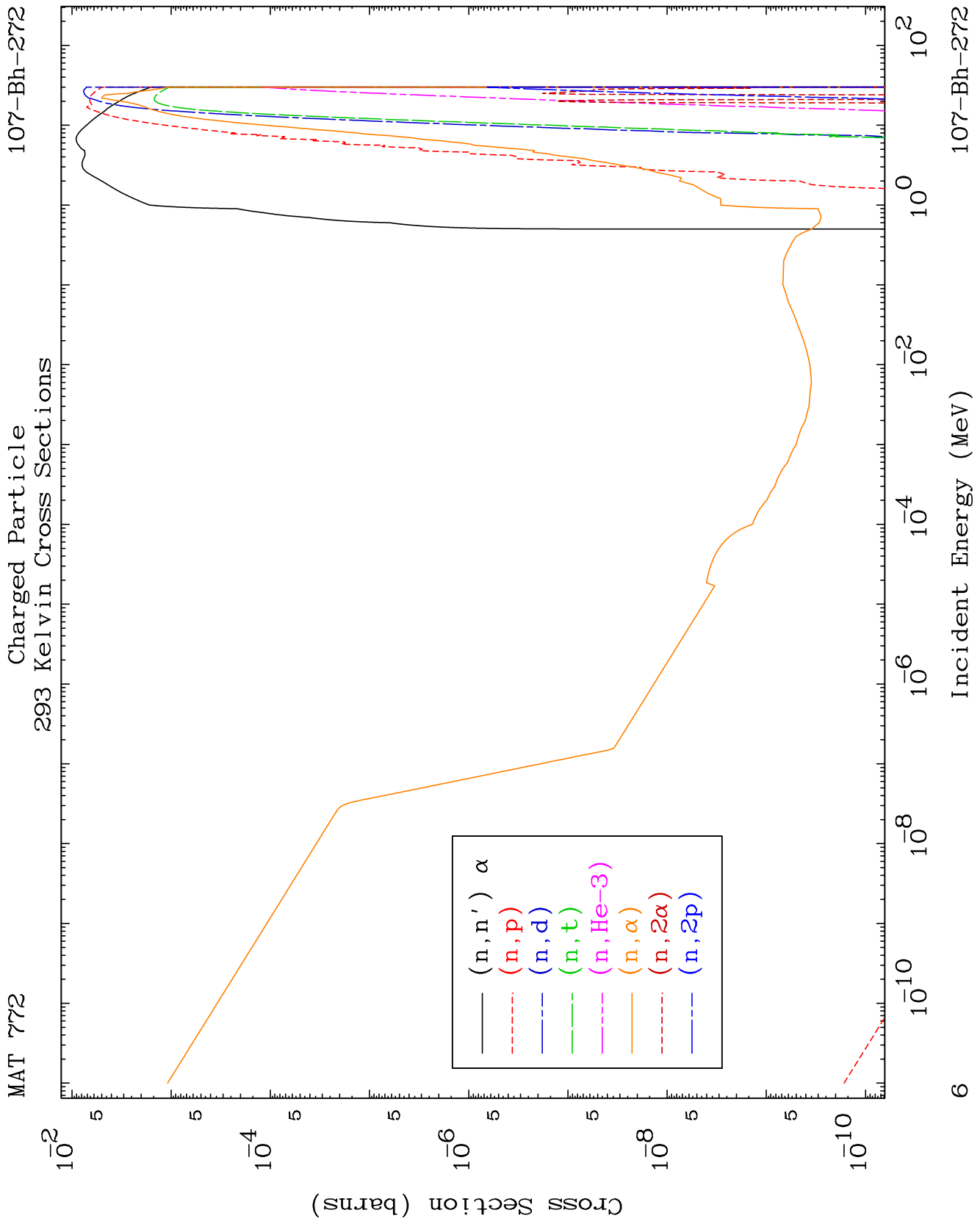


MAT 772

Charged Particle
293 Kelvin Cross Sections

107-Bh-272

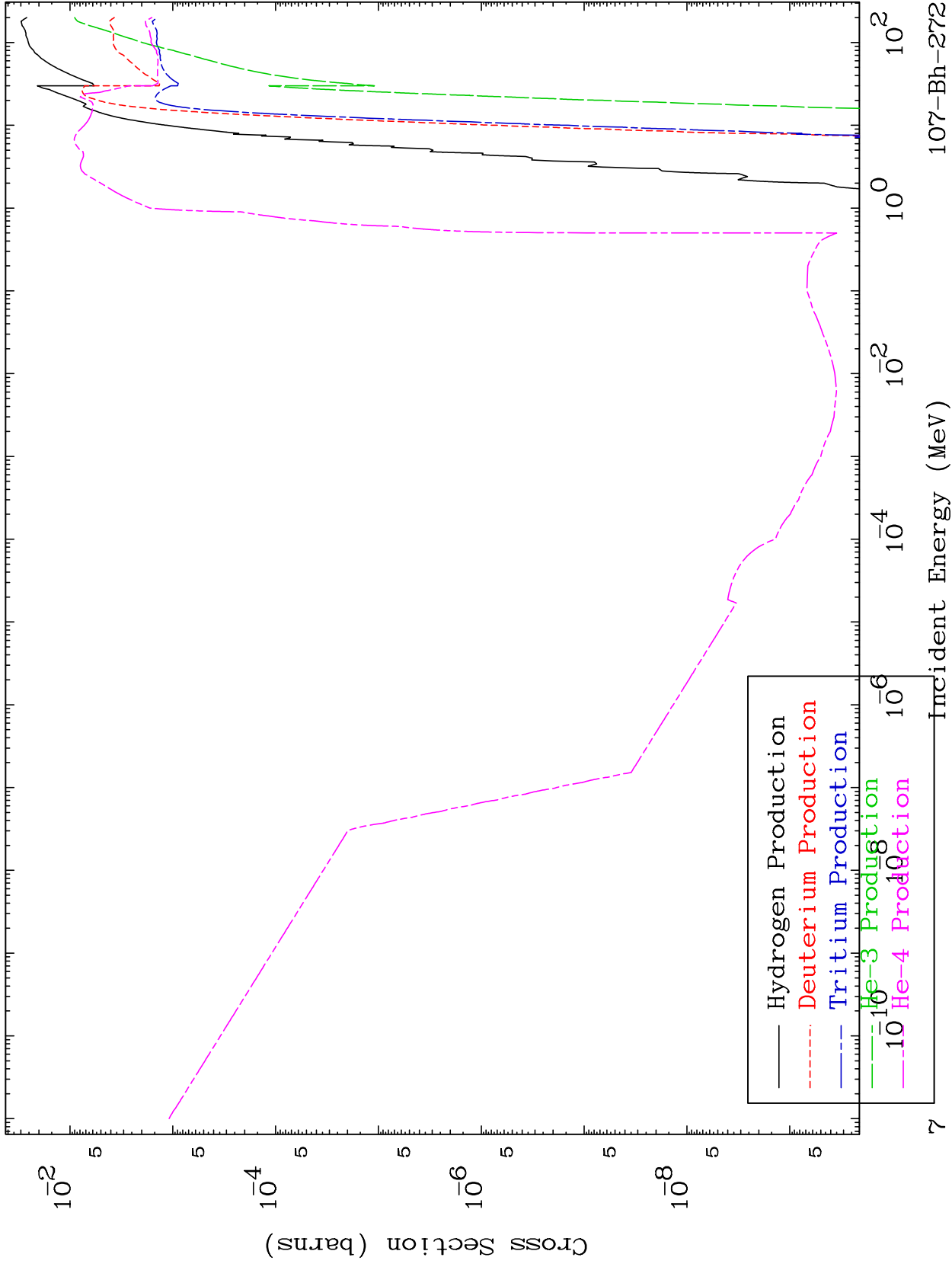




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Particle Production
293 Kelvin Cross Sections

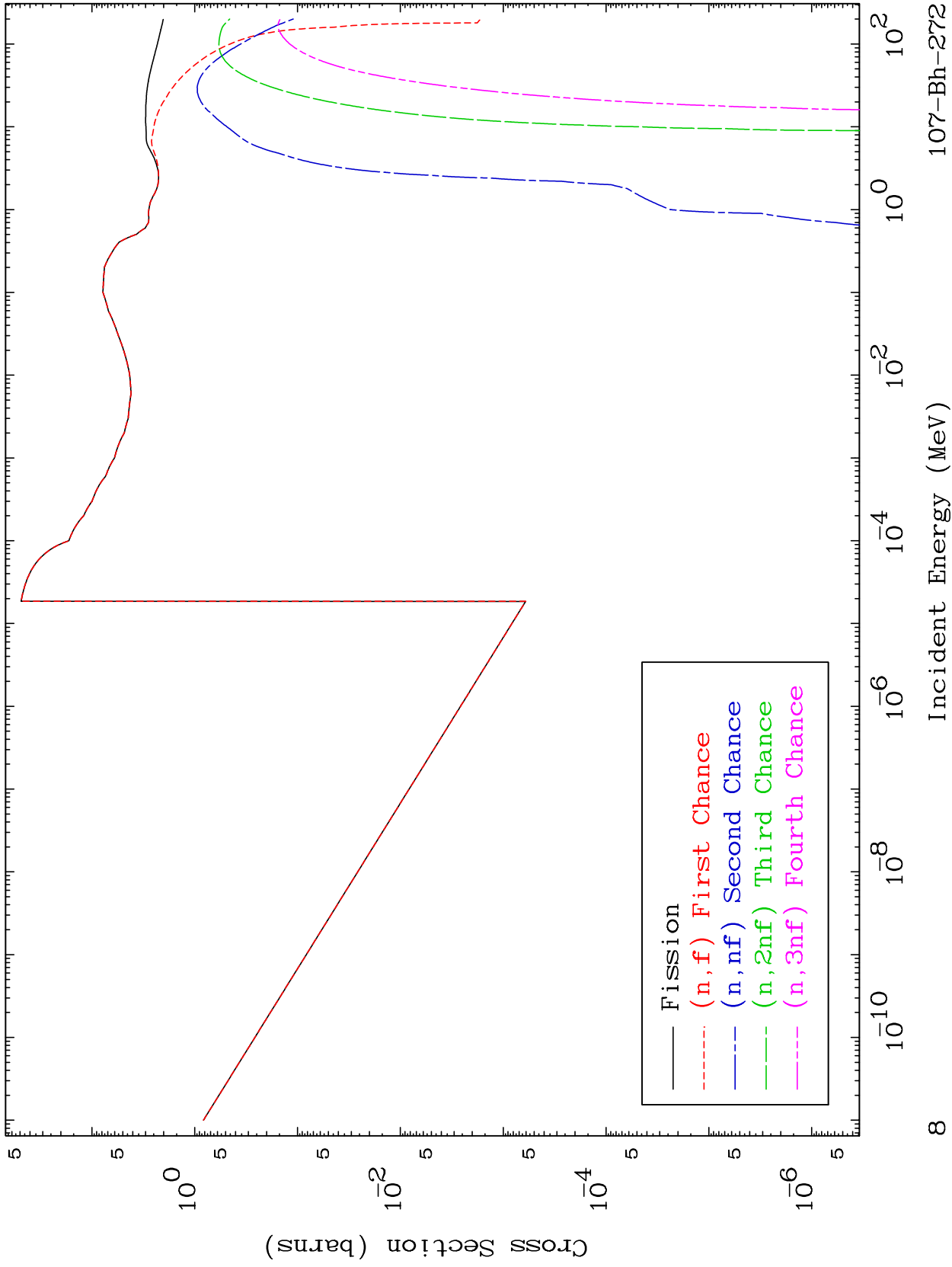
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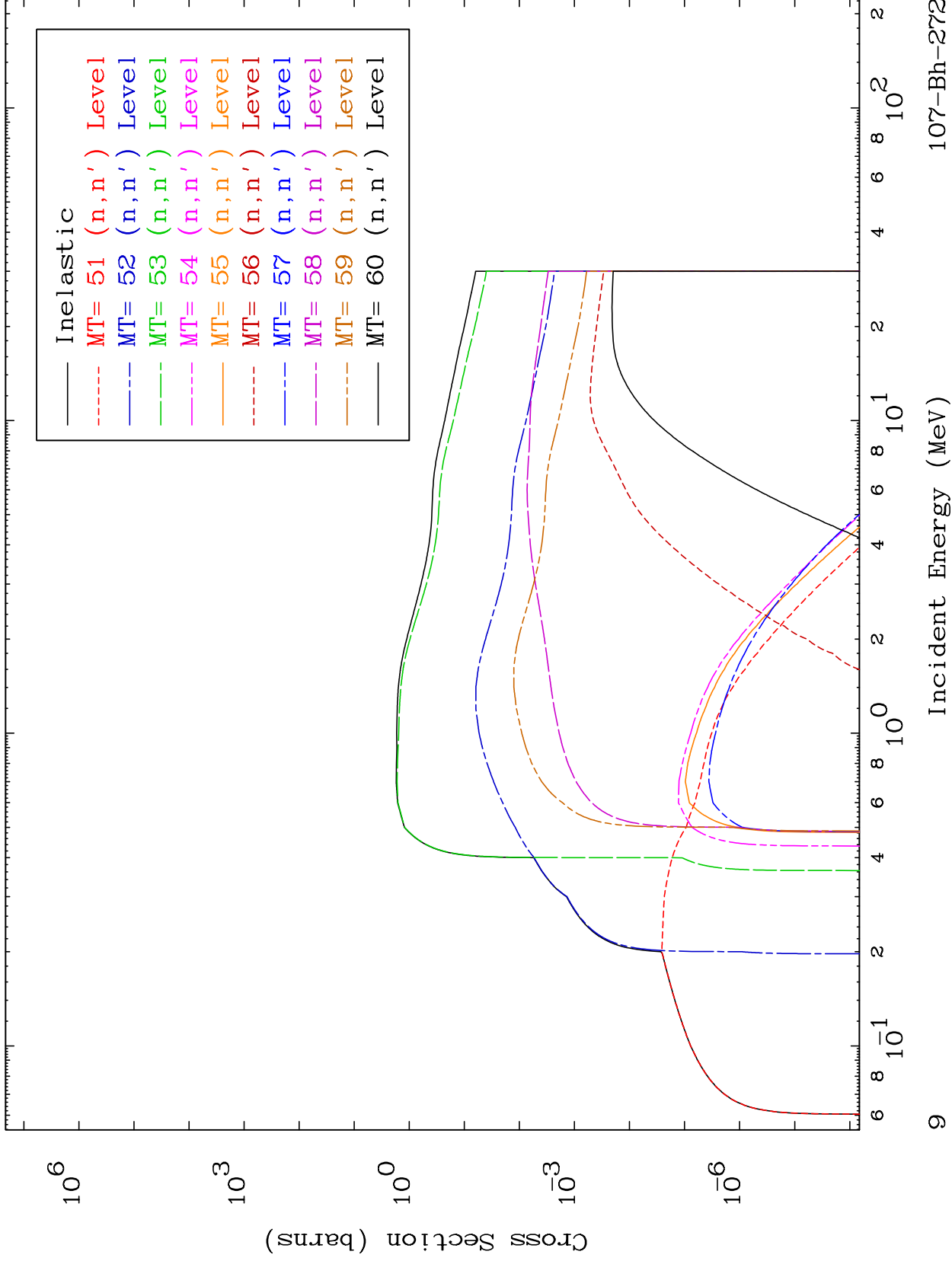


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Fission
293 Kelvin Cross Sections

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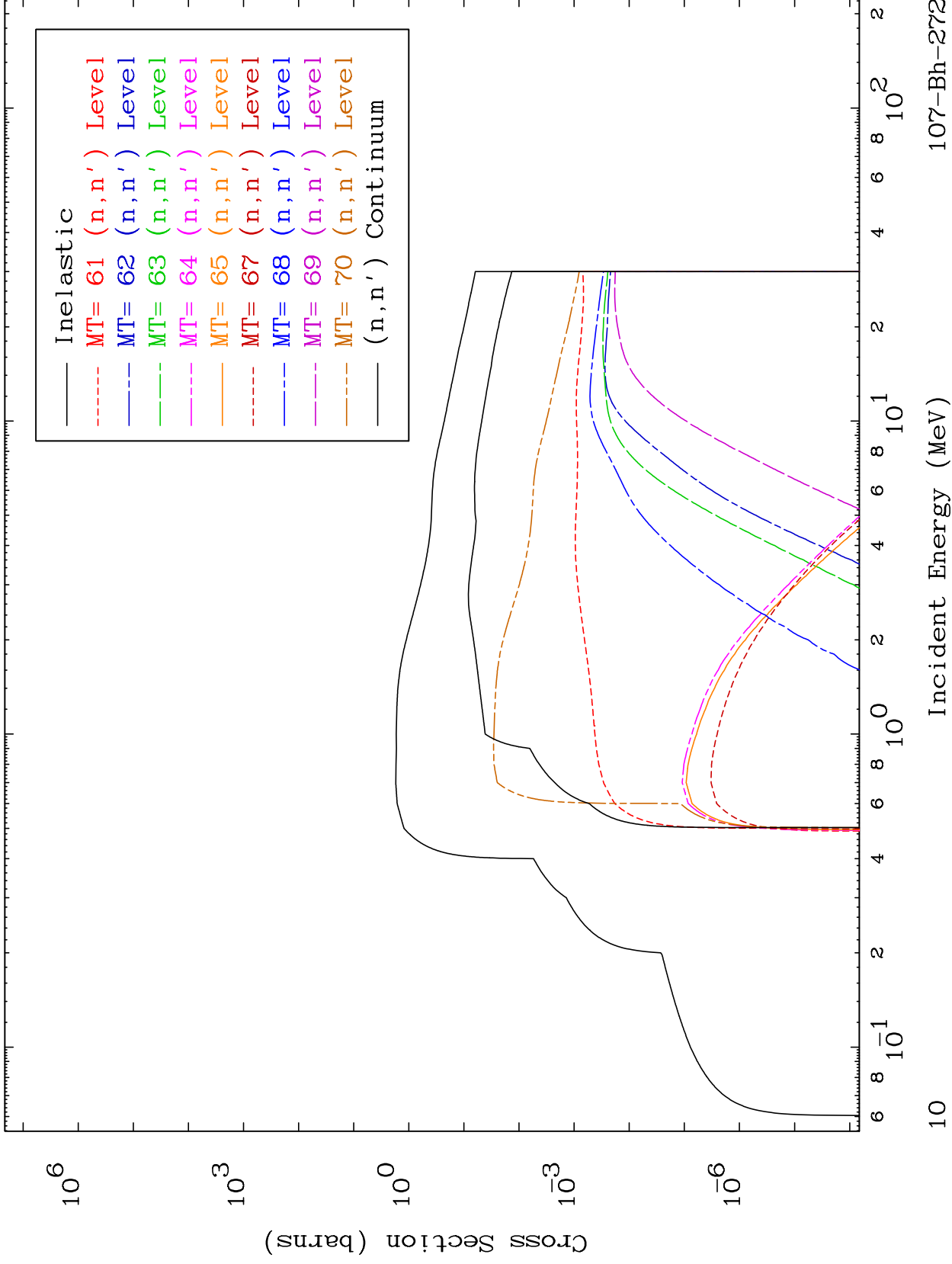


MAT 772

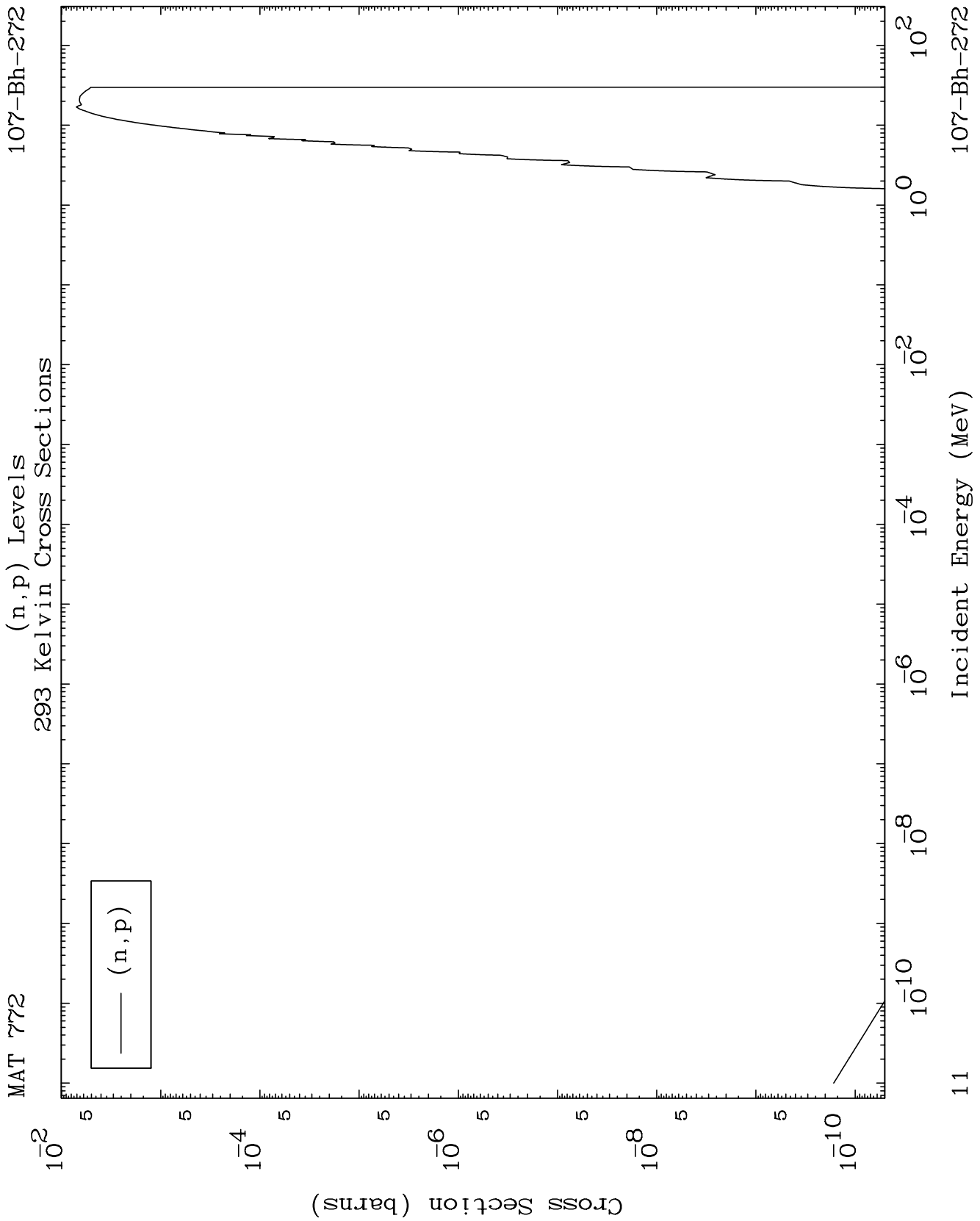
(n,n') Levels

293 Kelvin Cross Sections

107-Bh-272



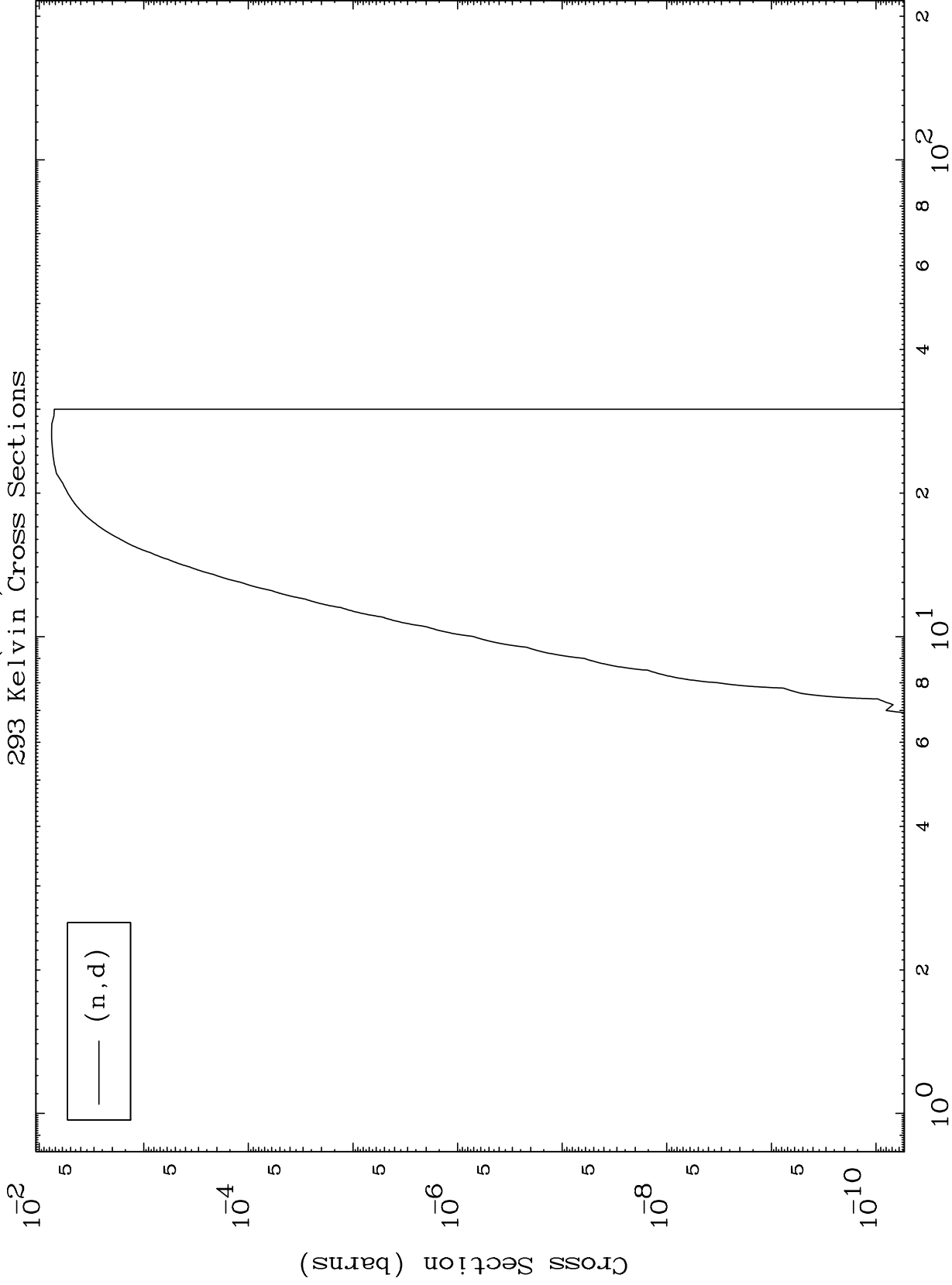
10



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(n,d) Levels
293 Kelvin Cross Sections

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(n,d)

Incident Energy (MeV)

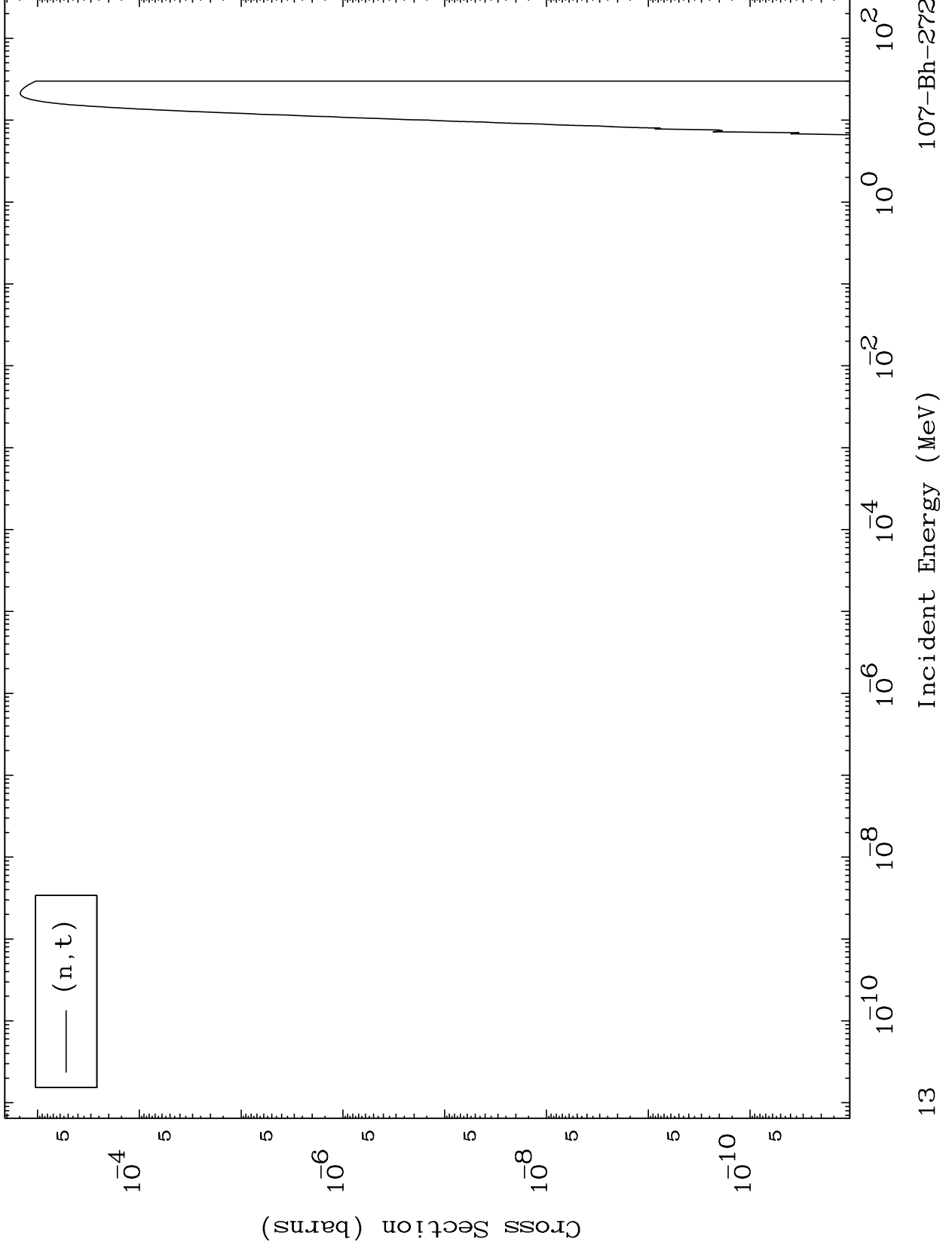
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12

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(n,t) Levels
293 Kelvin Cross Sections

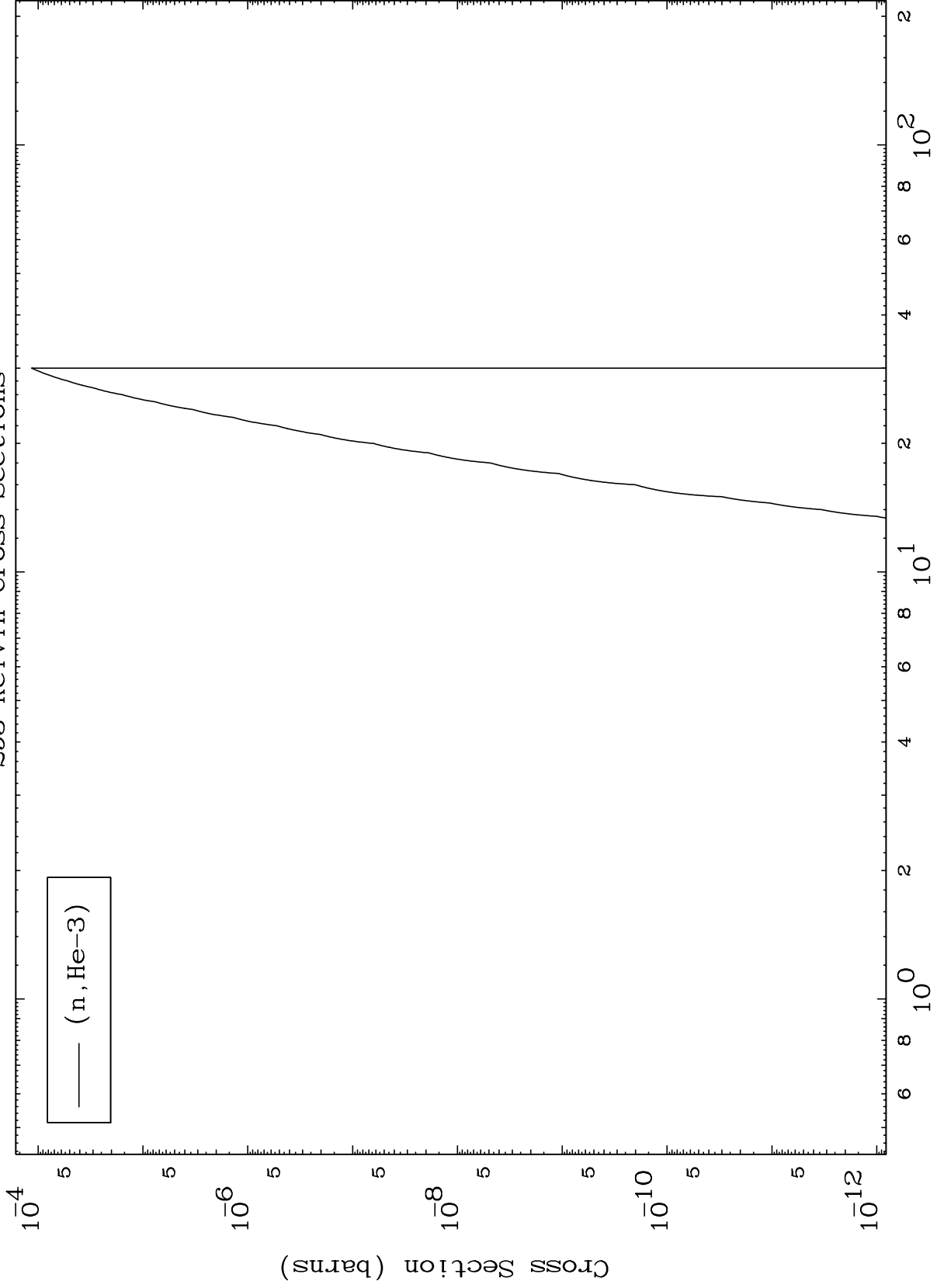
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(n,He3) Levels
293 Kelvin Cross Sections

107-Bh-272



14

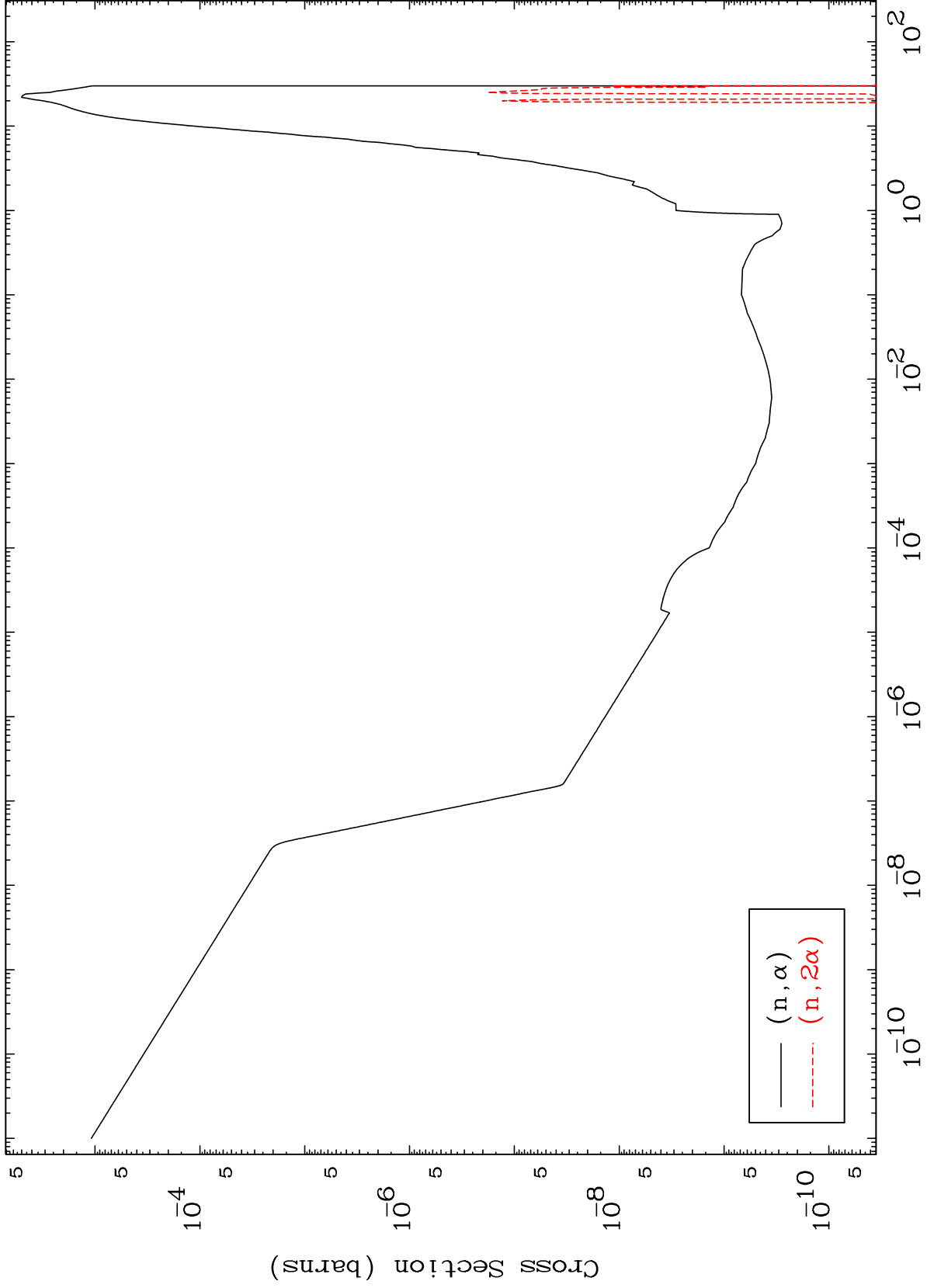
Incident Energy (MeV)

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(n, α) Levels
293 Kelvin Cross Sections

107-Bh-272



15

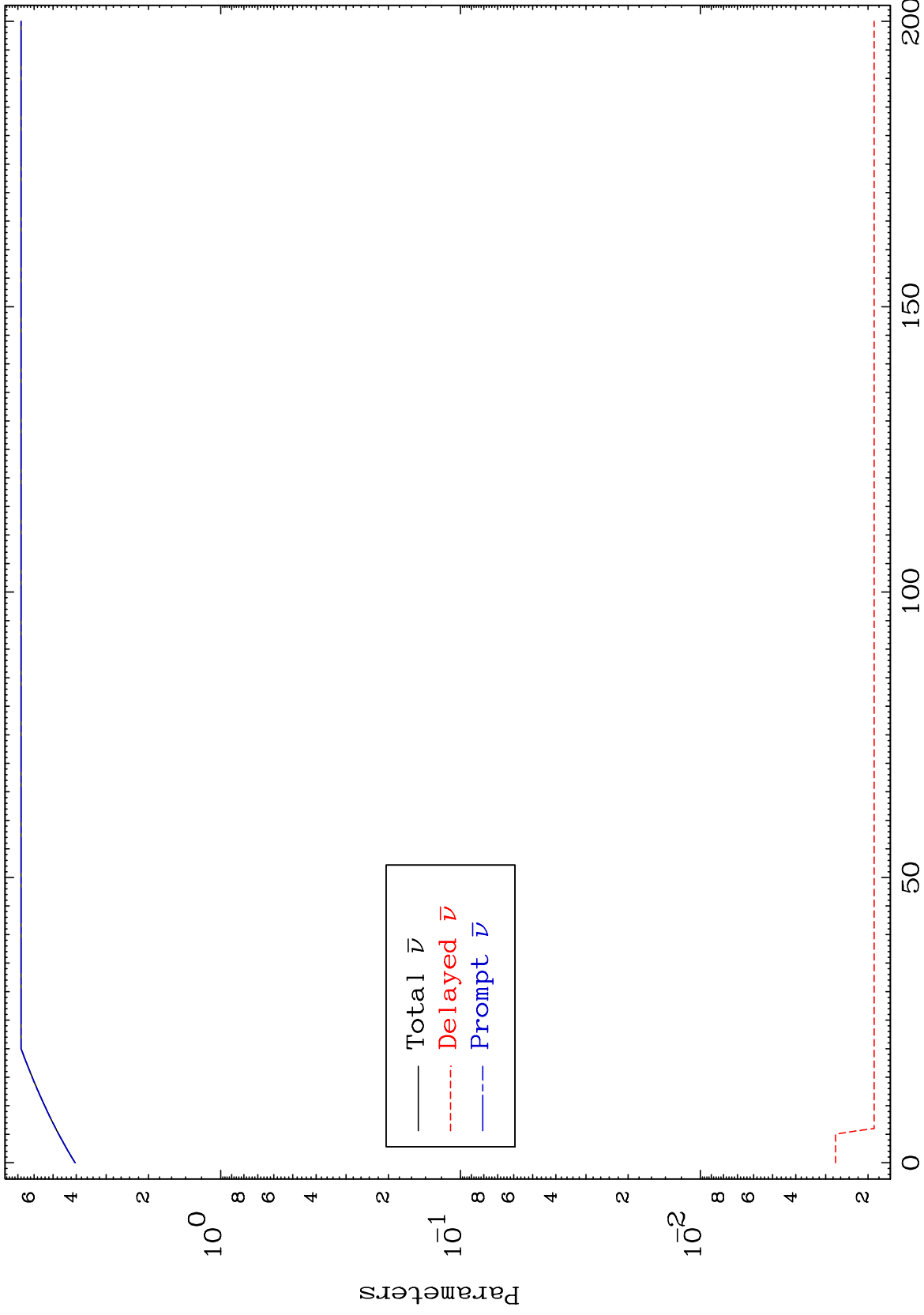
Incident Energy (MeV)

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Energy Release
Parameters

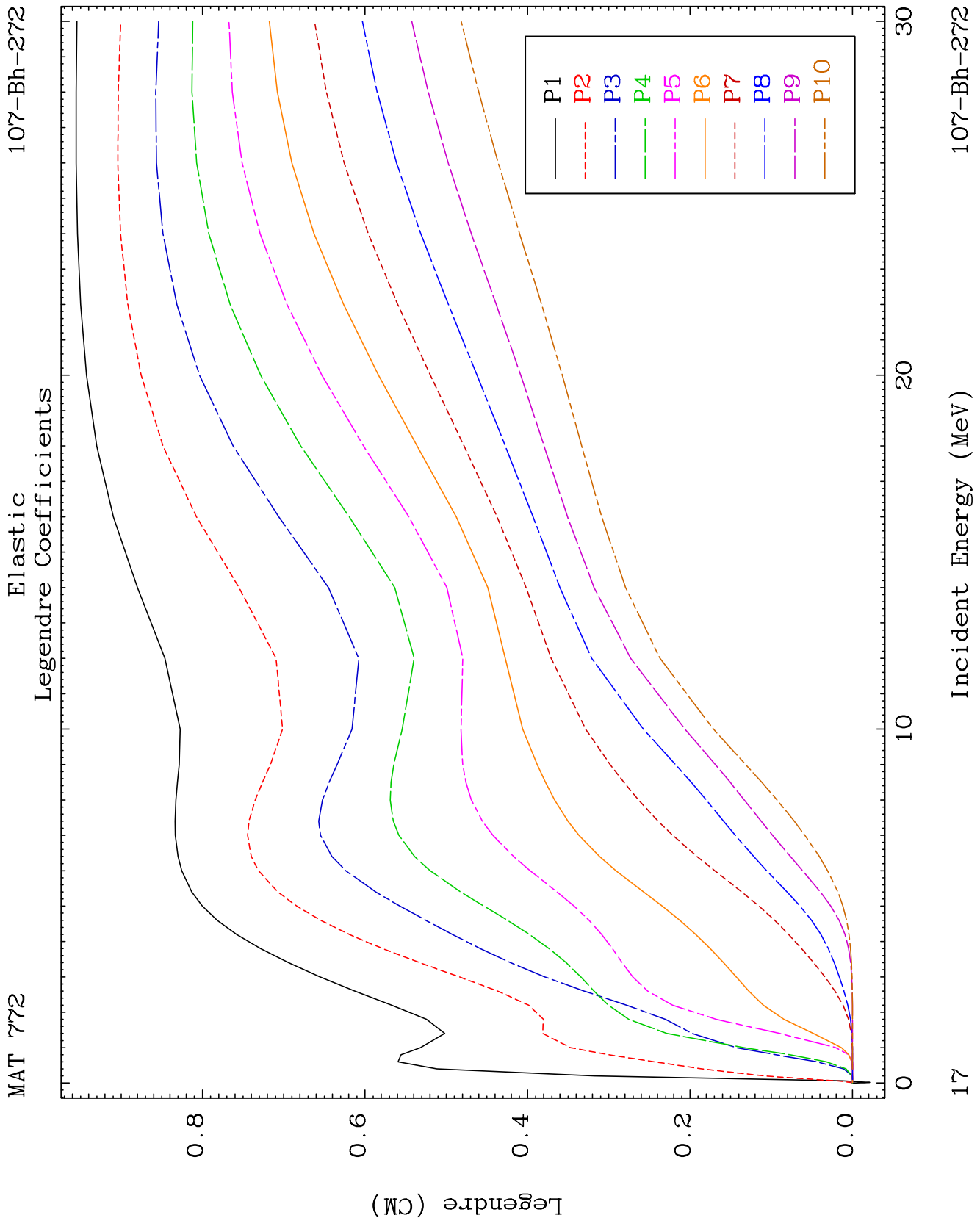
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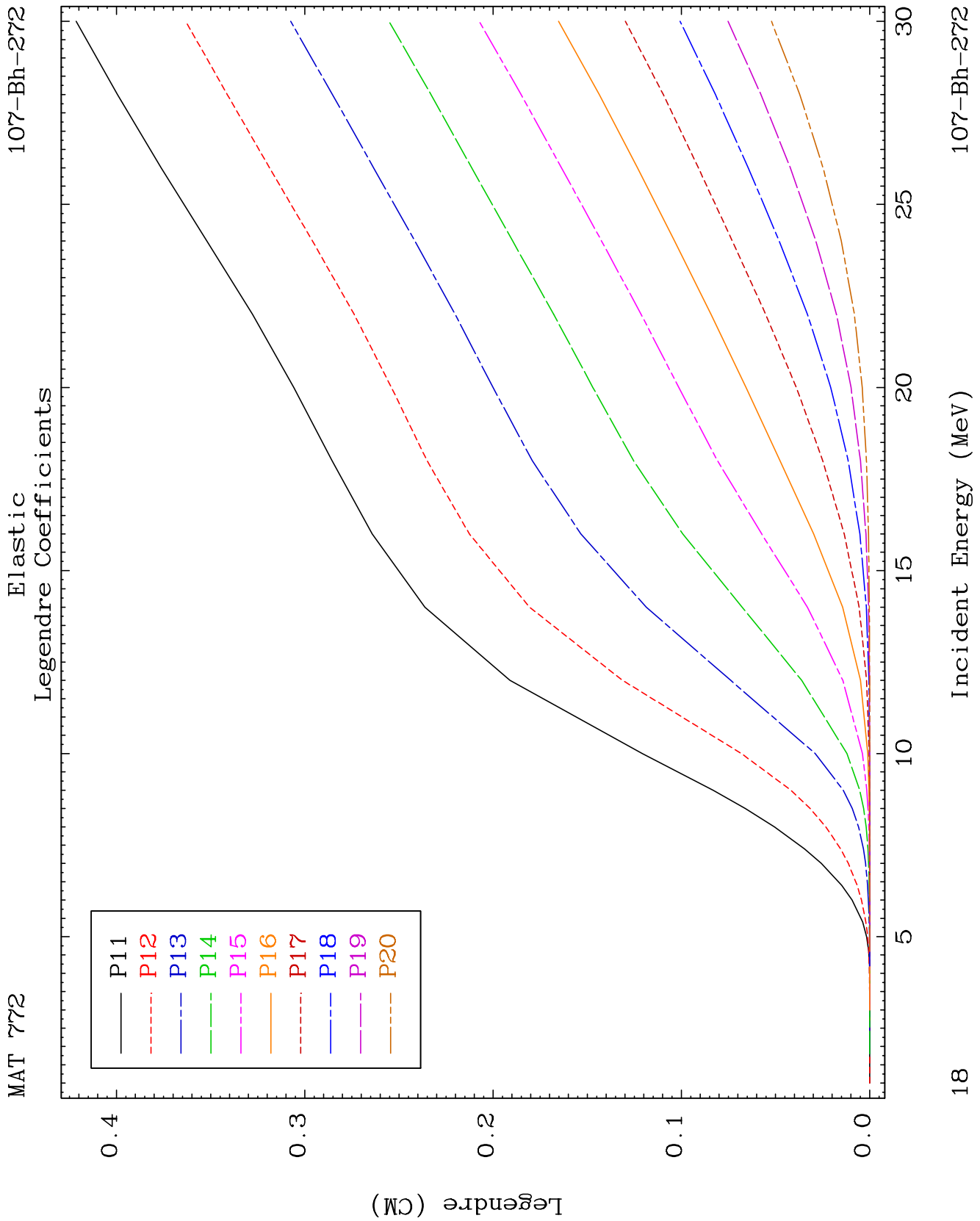


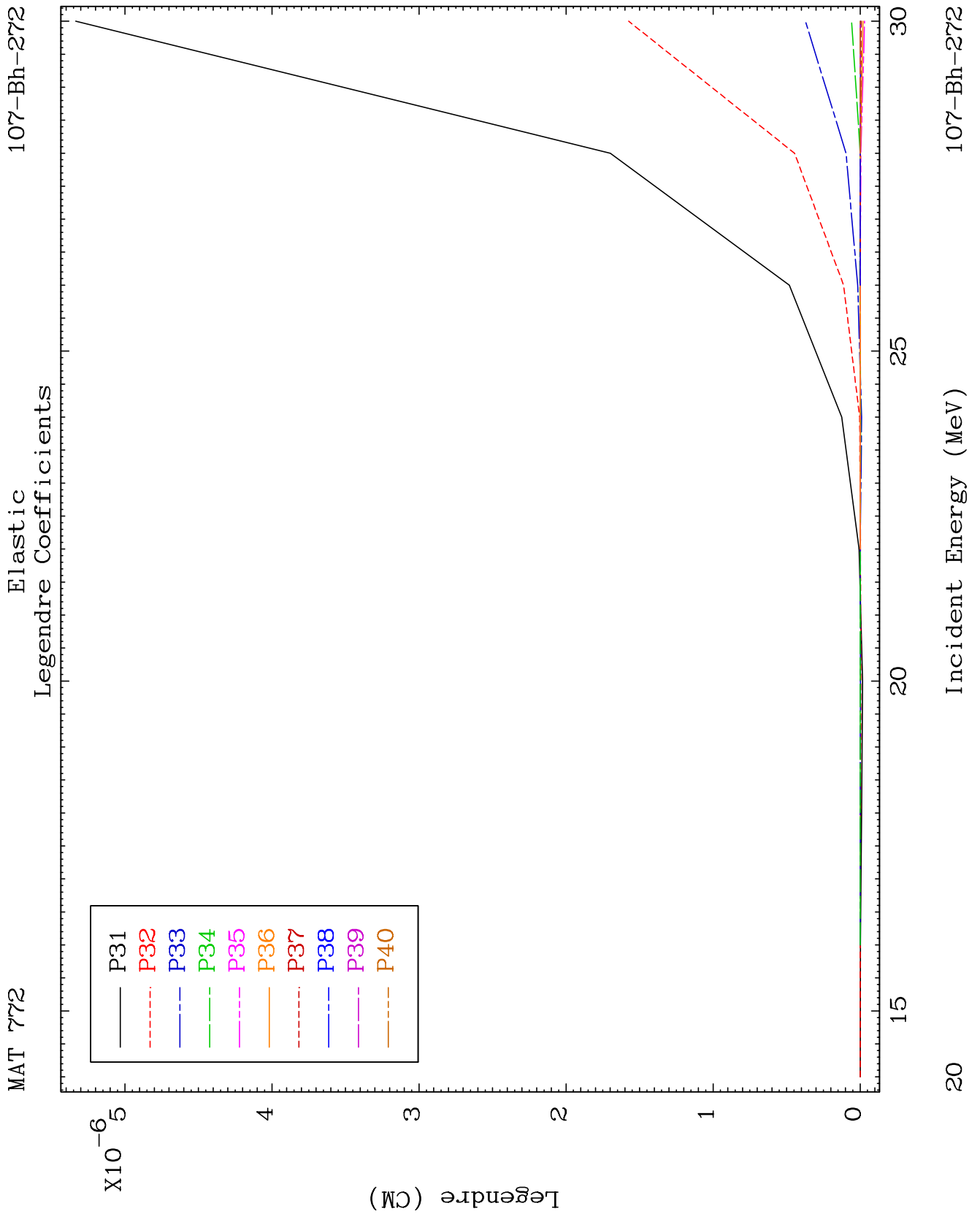
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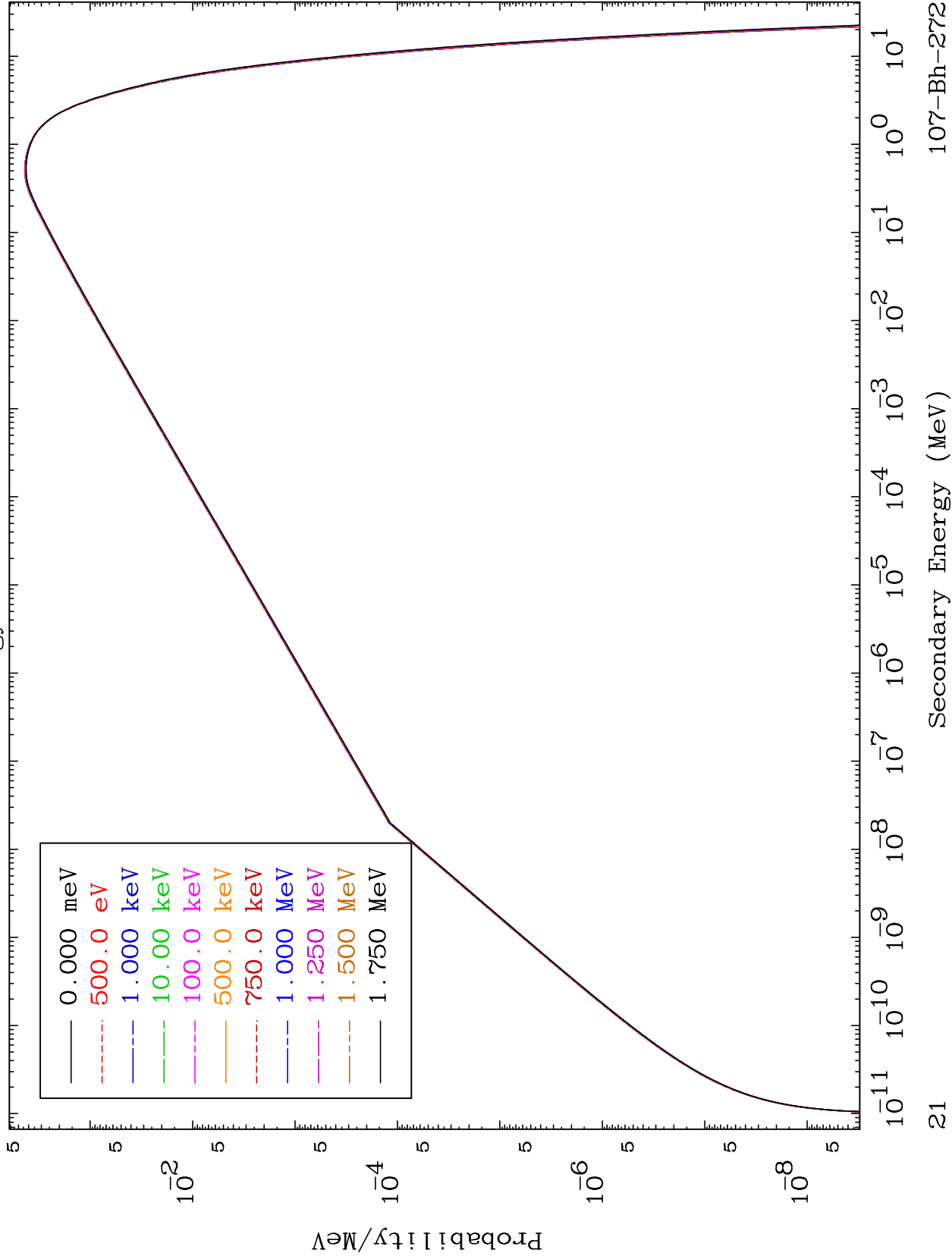
Incident Energy (MeV)

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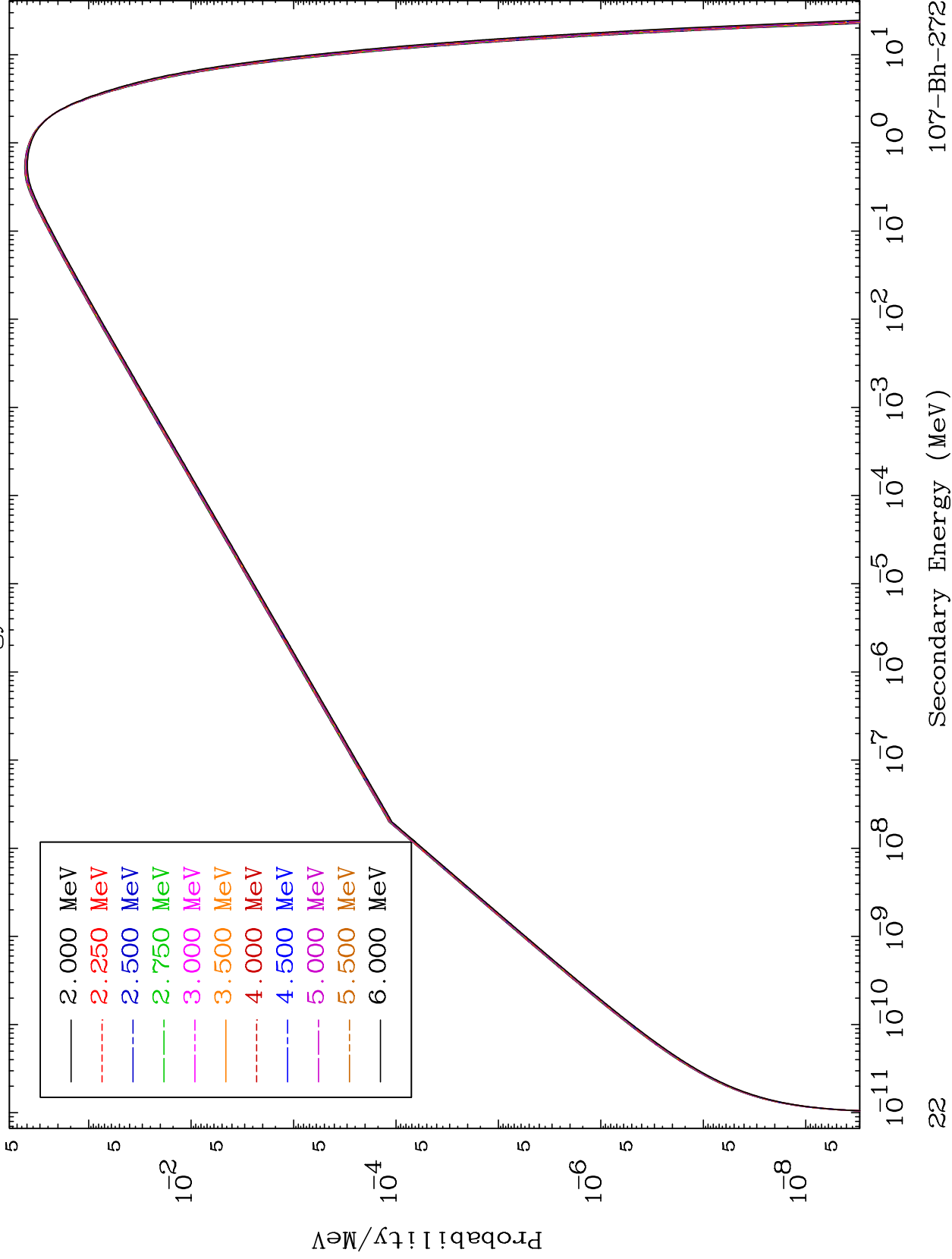




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Fission Energy Distributions

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107-Bh-272

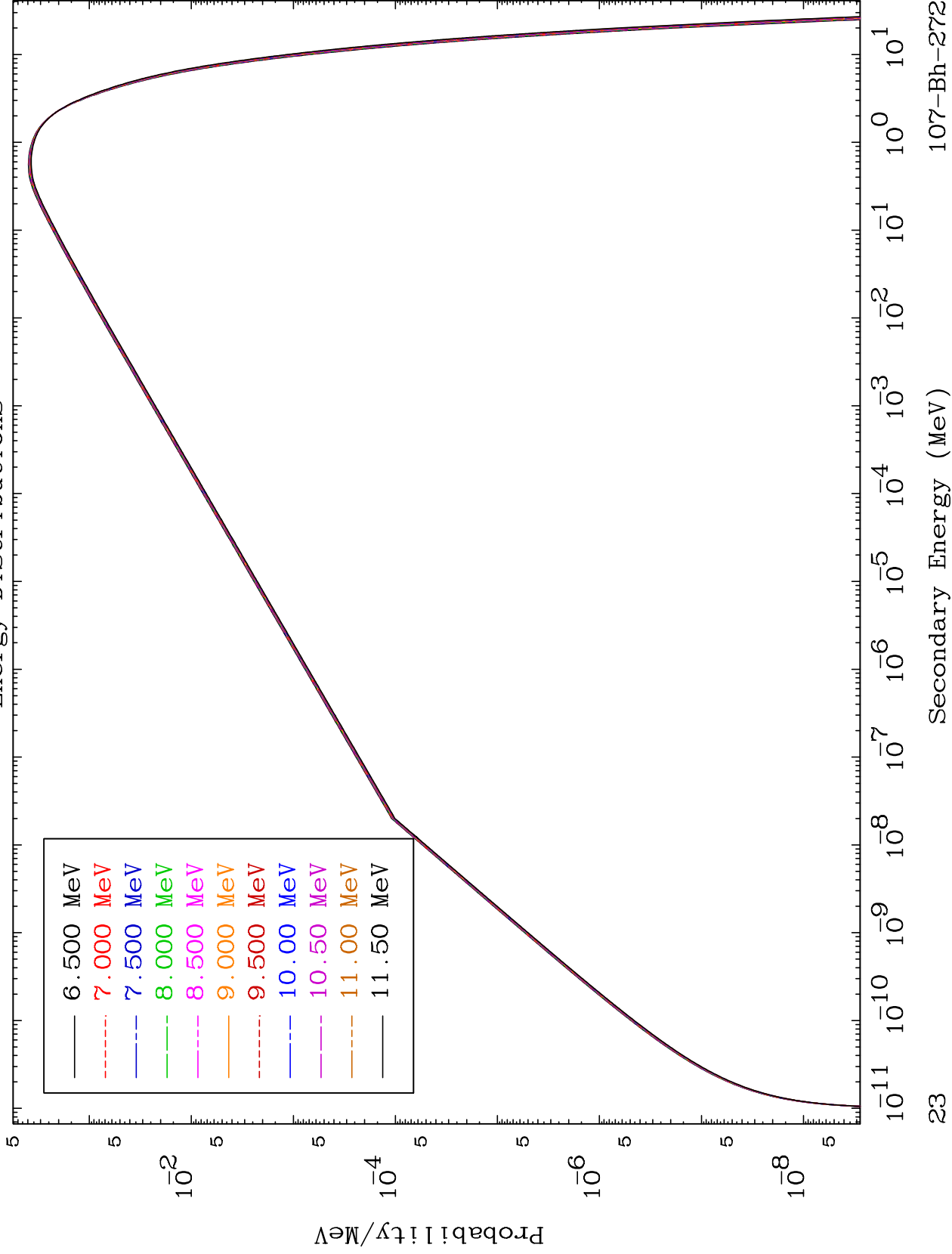
Secondary Energy (MeV)

22

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Fission Energy Distributions

107-Bh-272

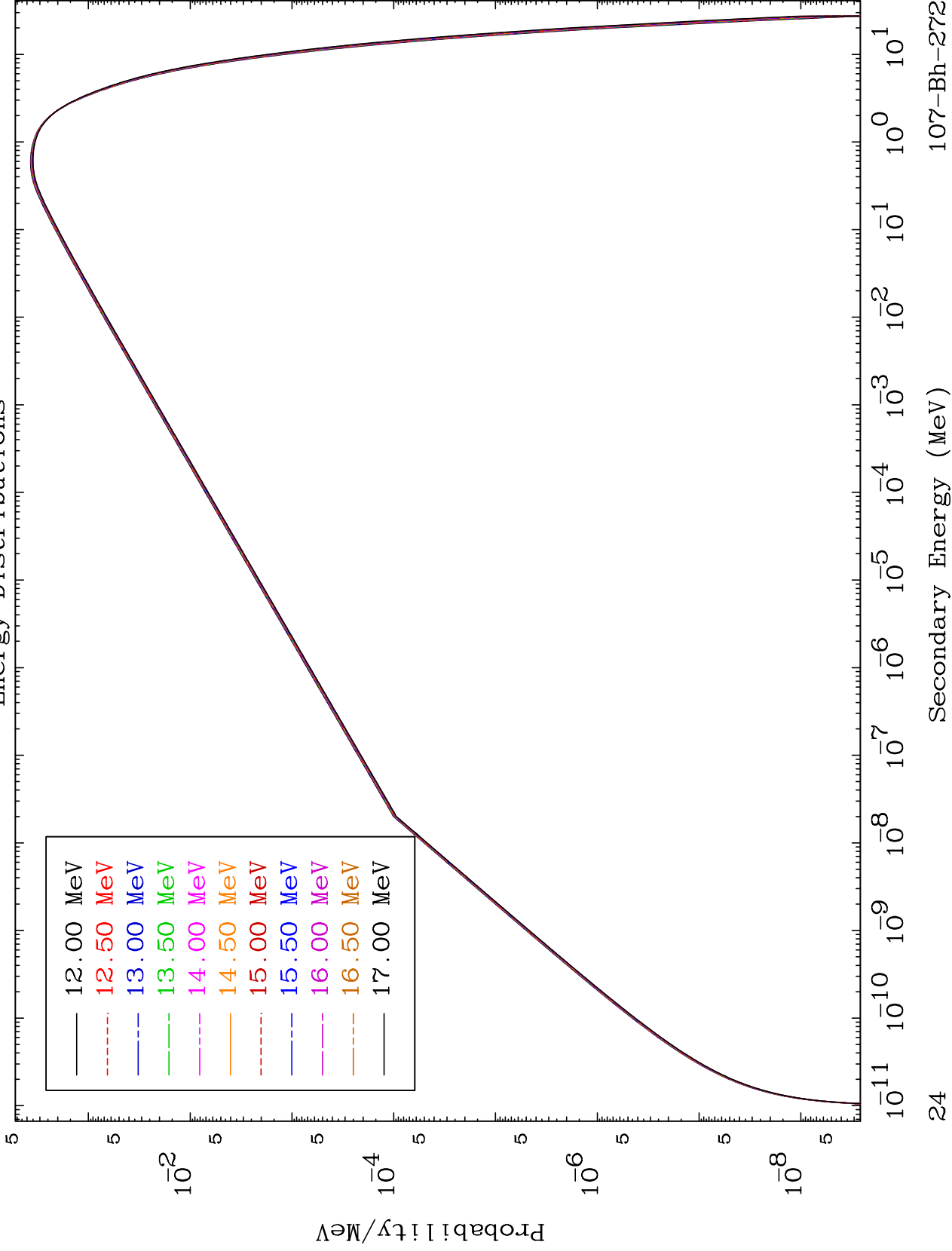


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Fission Energy Distributions

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Secondary Energy (MeV)

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MAT 772

Fission Energy Distributions

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