

Program EVALPLOT
(Version 2021-1)

by

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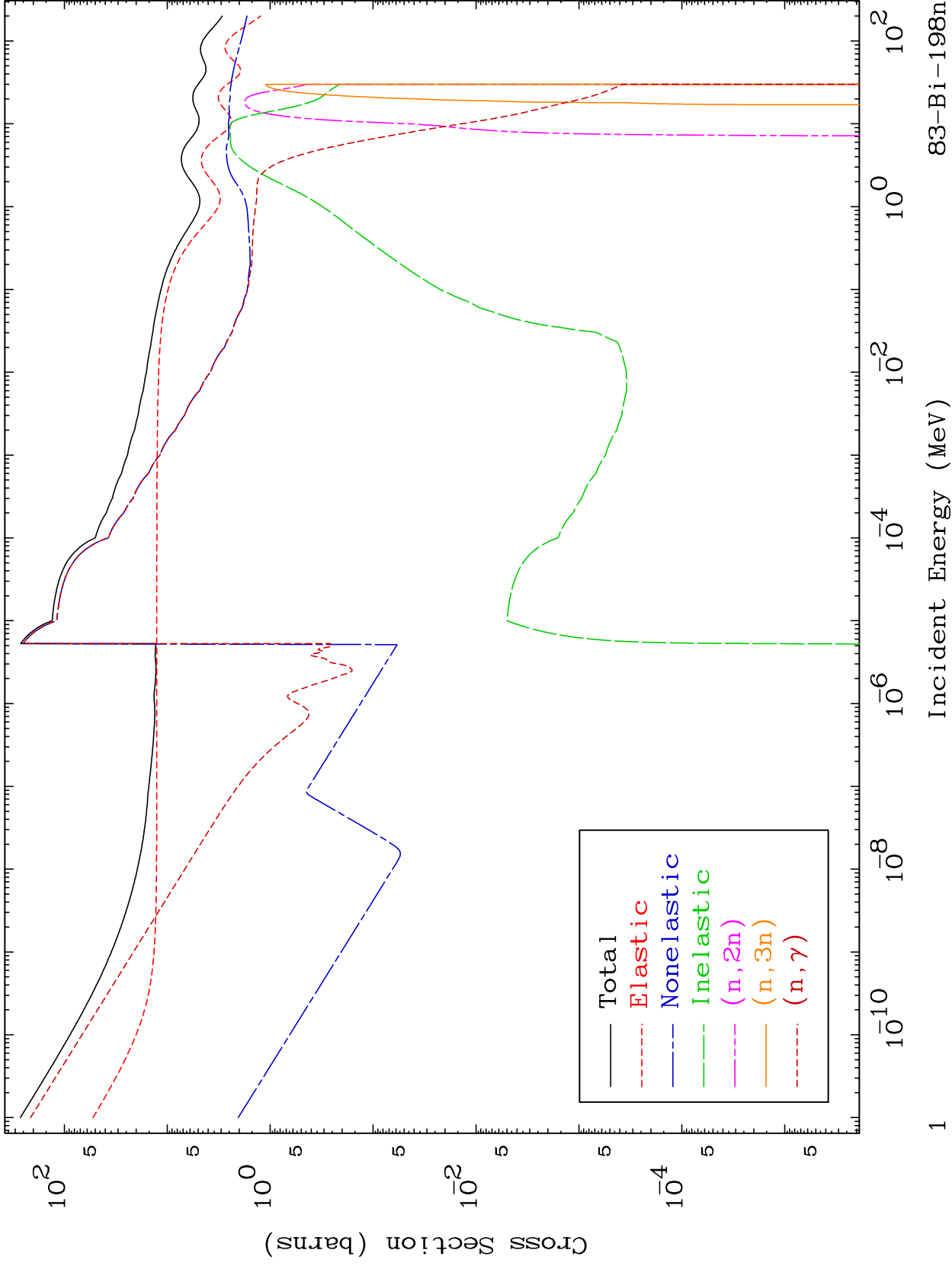
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Press Mouse Button to Start

MAT 8294

Neutron Major
293 Kelvin Cross Sections

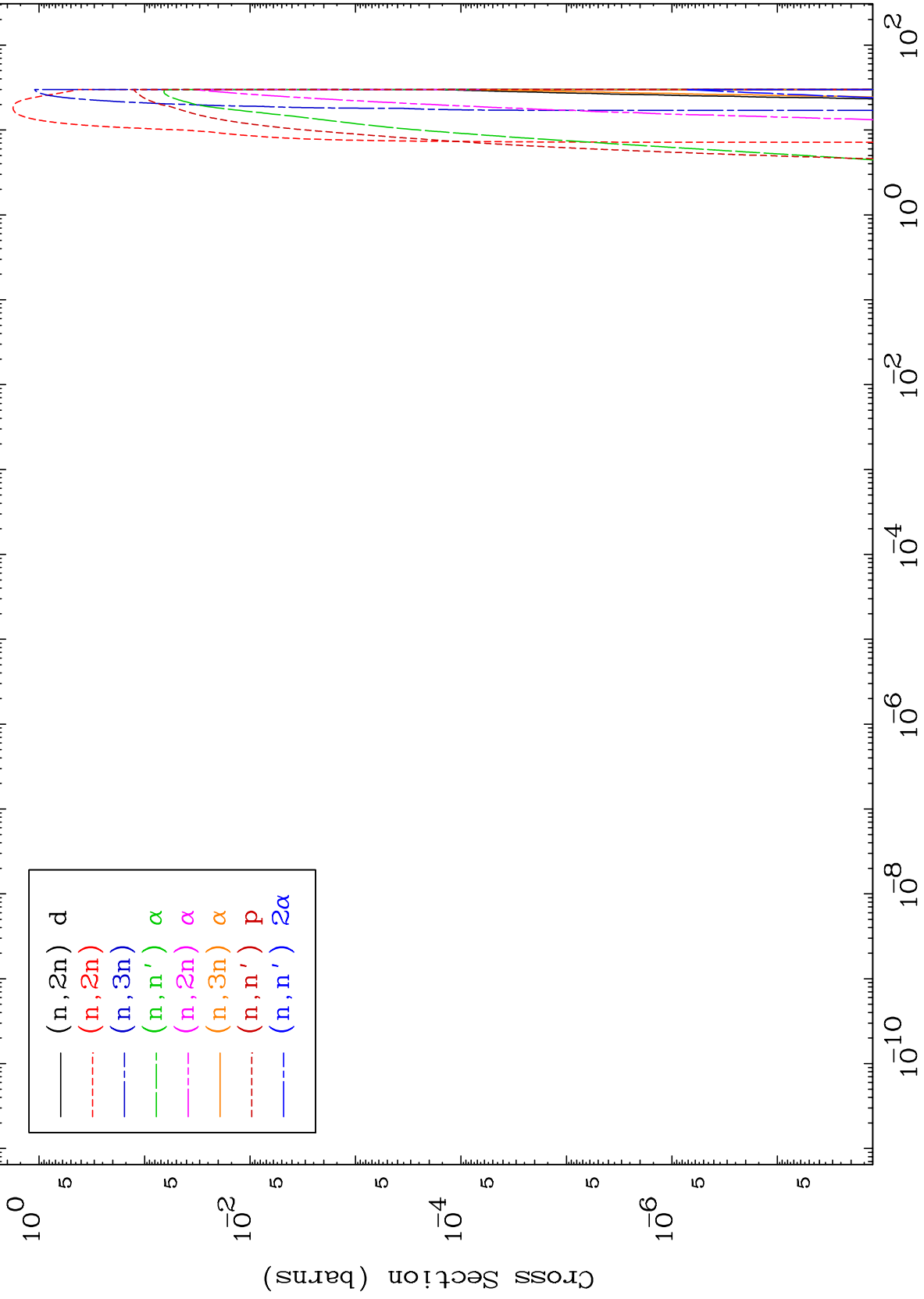
83-Bi-198n



MAT 8294

Neutron Absorption
293 Kelvin Cross Sections

83-Bi-198n



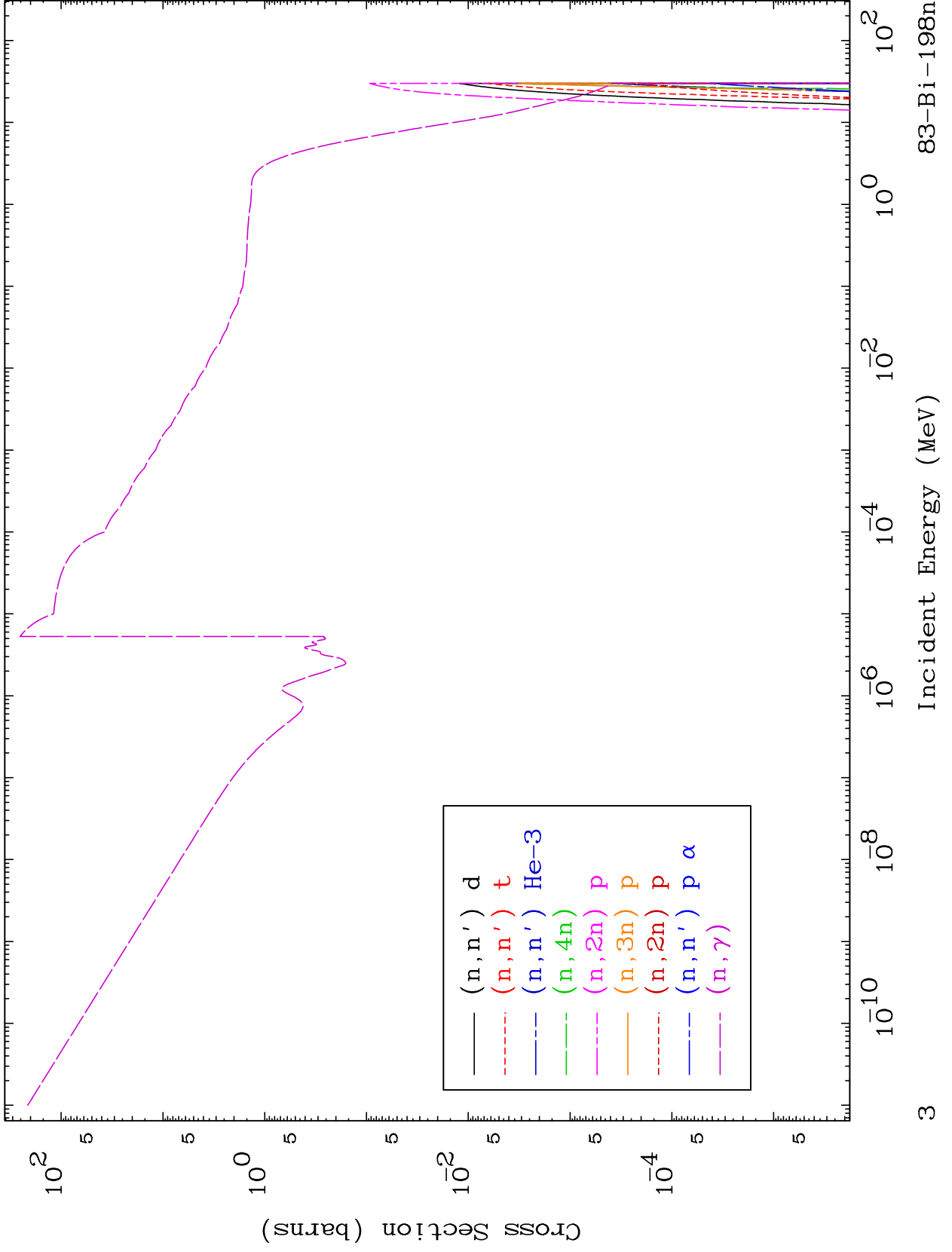
83-Bi-198n

Incident Energy (MeV)

MAT 8294

Neutron Absorption
293 Kelvin Cross Sections

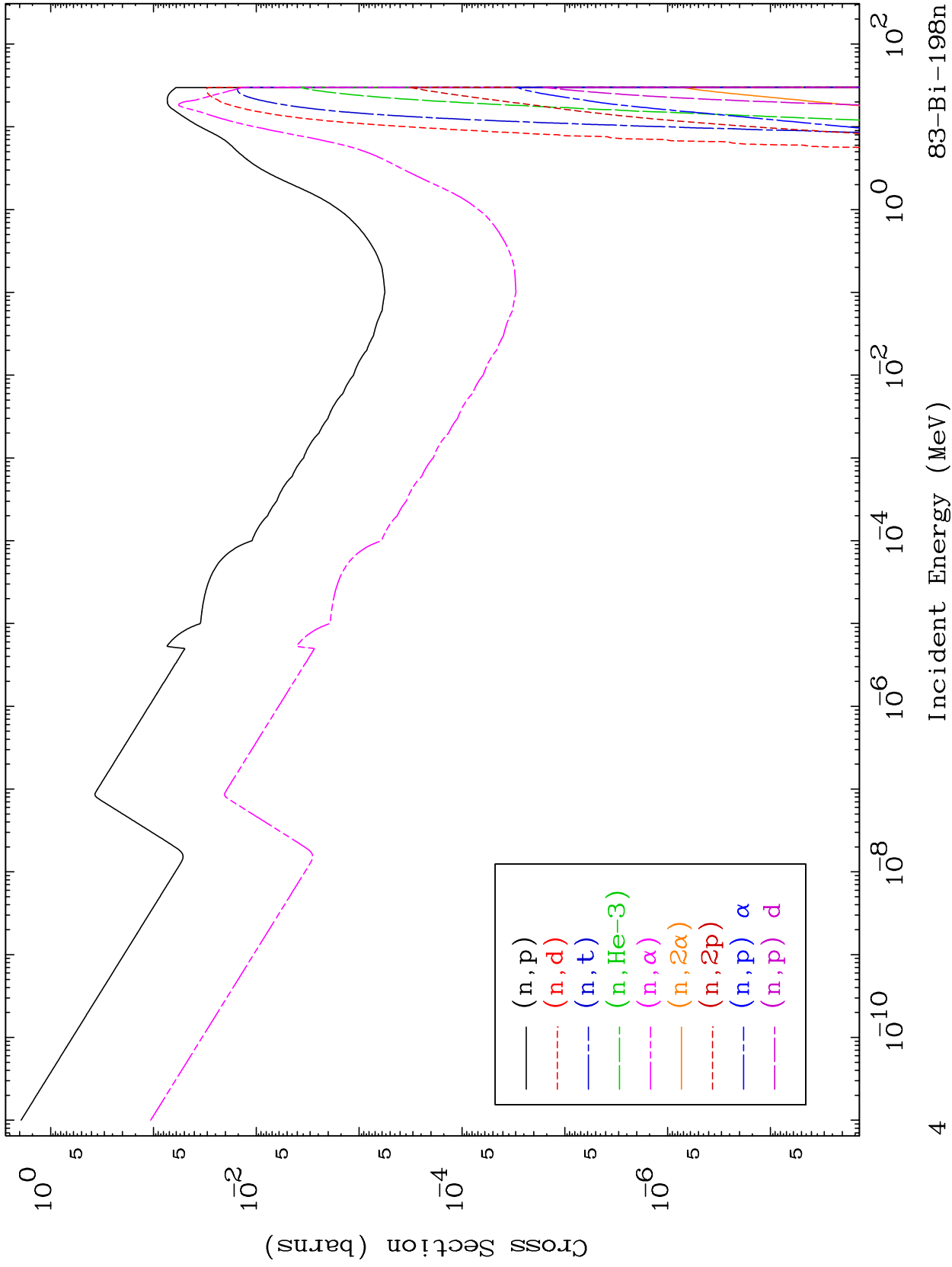
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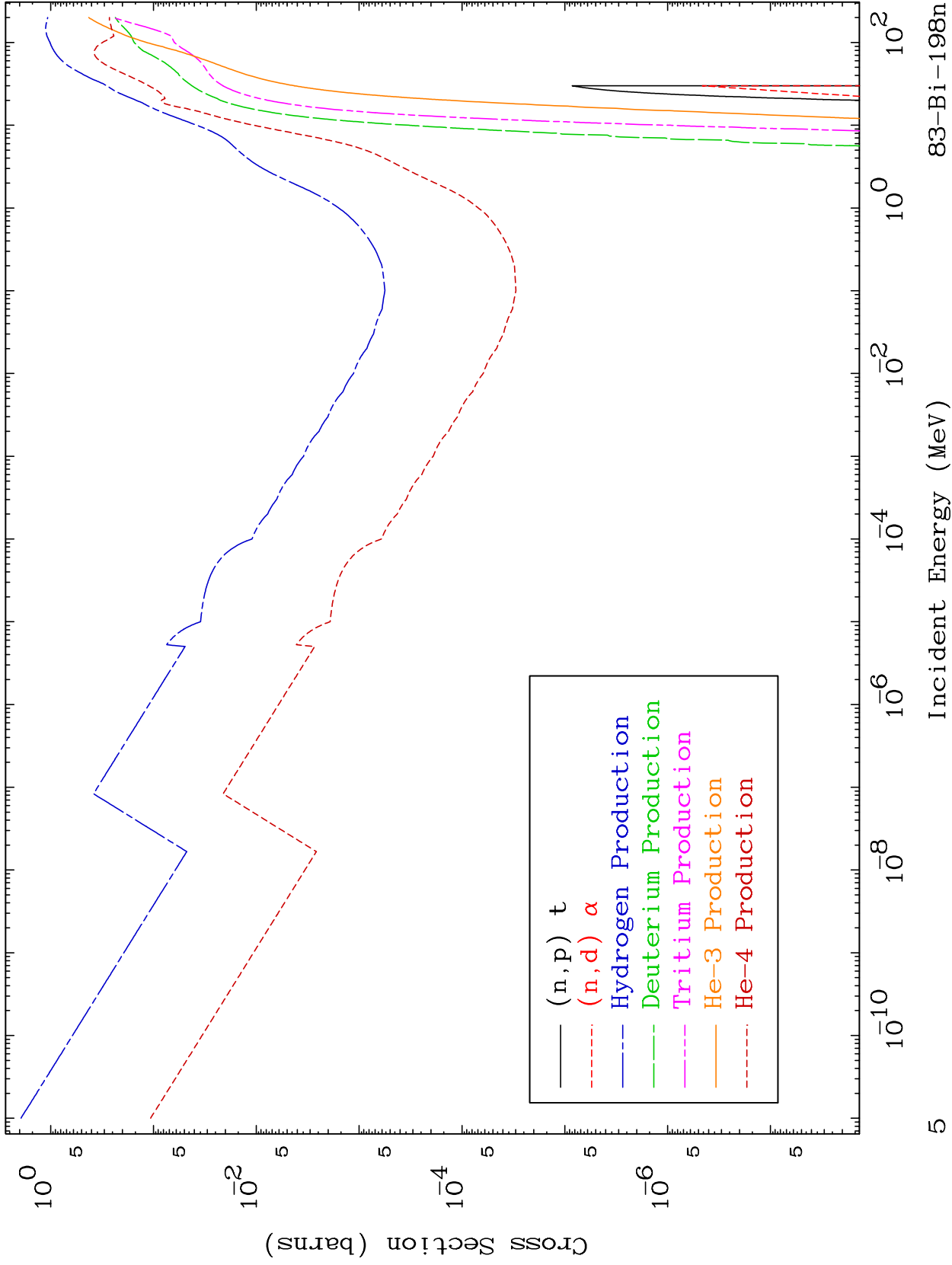


MAT 8294

Neutron Absorption
293 Kelvin Cross Sections

83-Bi-198n

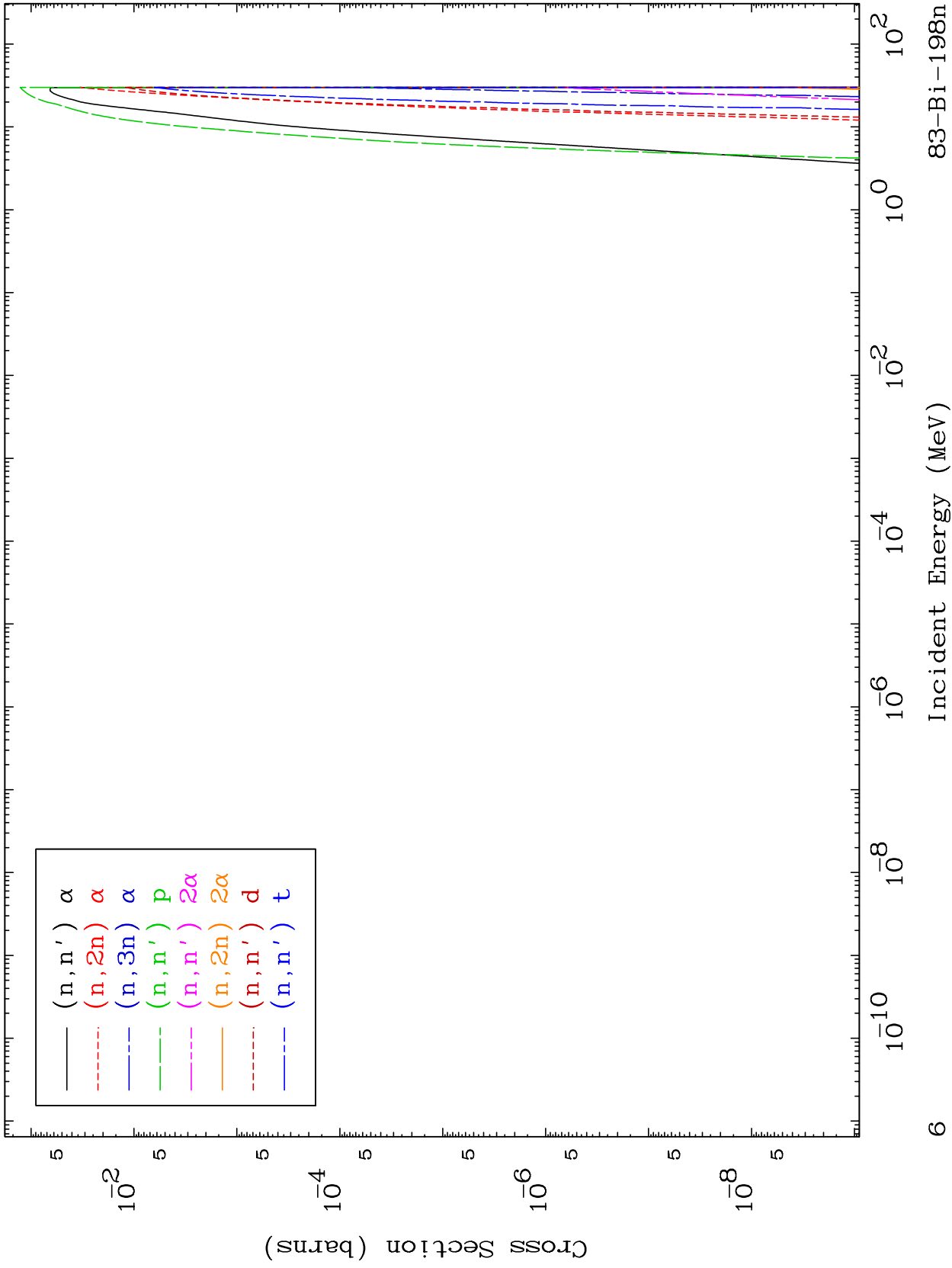




MAT 8294

Charged Particle
293 Kelvin Cross Sections

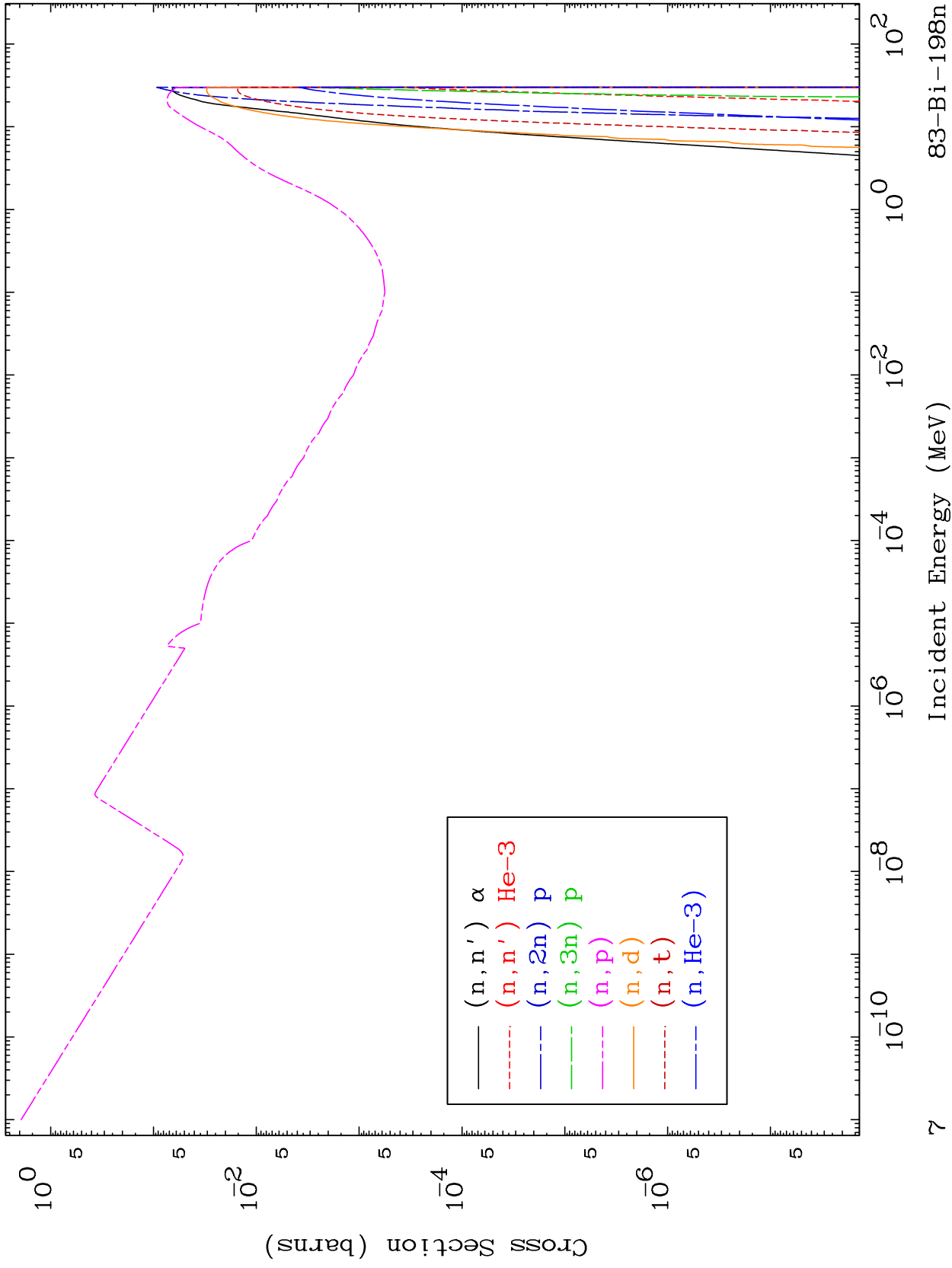
83-Bi-198n



MAT 8294

Charged Particle
293 Kelvin Cross Sections

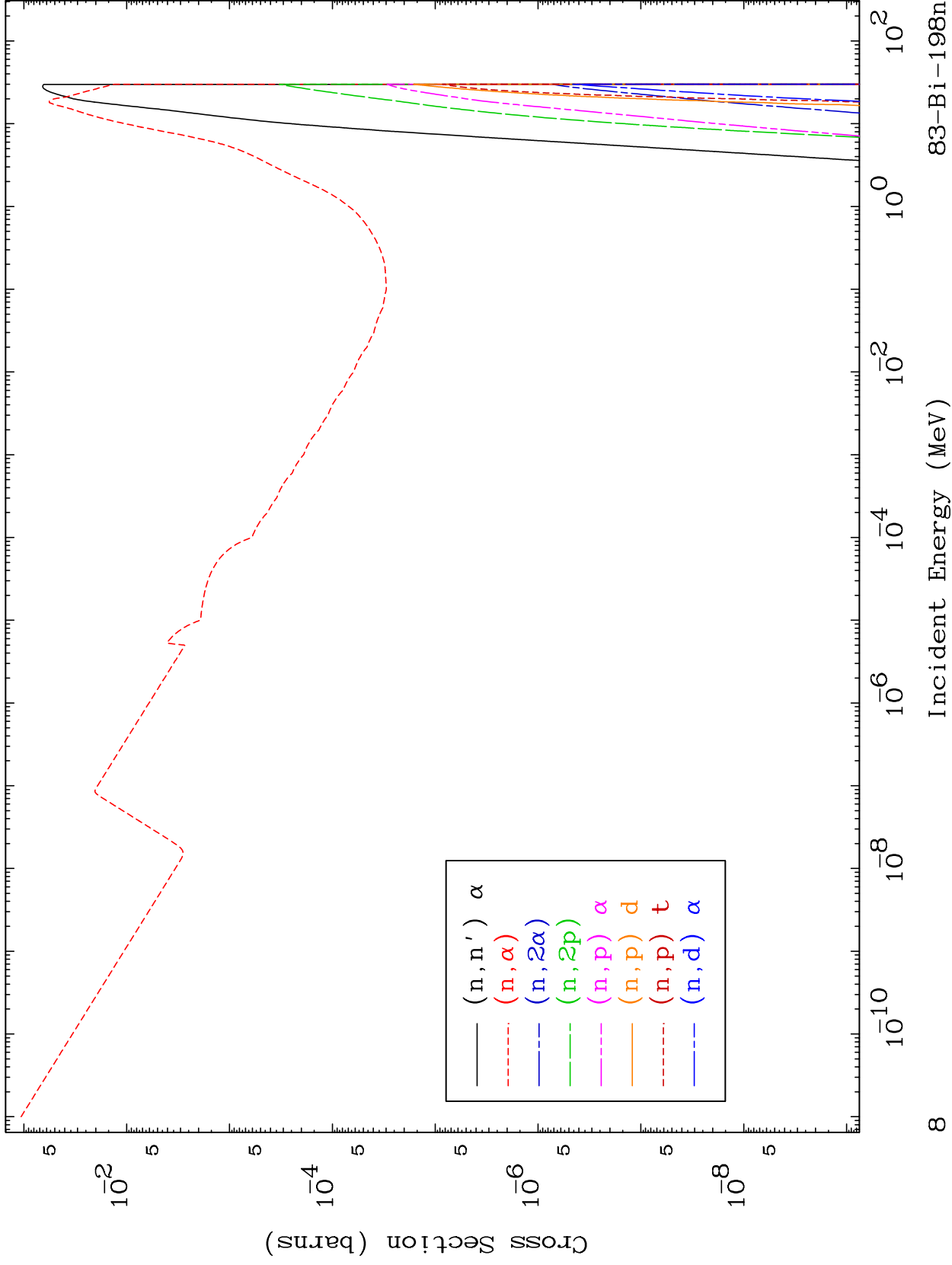
83-Bi-198n



MAT 8294

Charged Particle
293 Kelvin Cross Sections

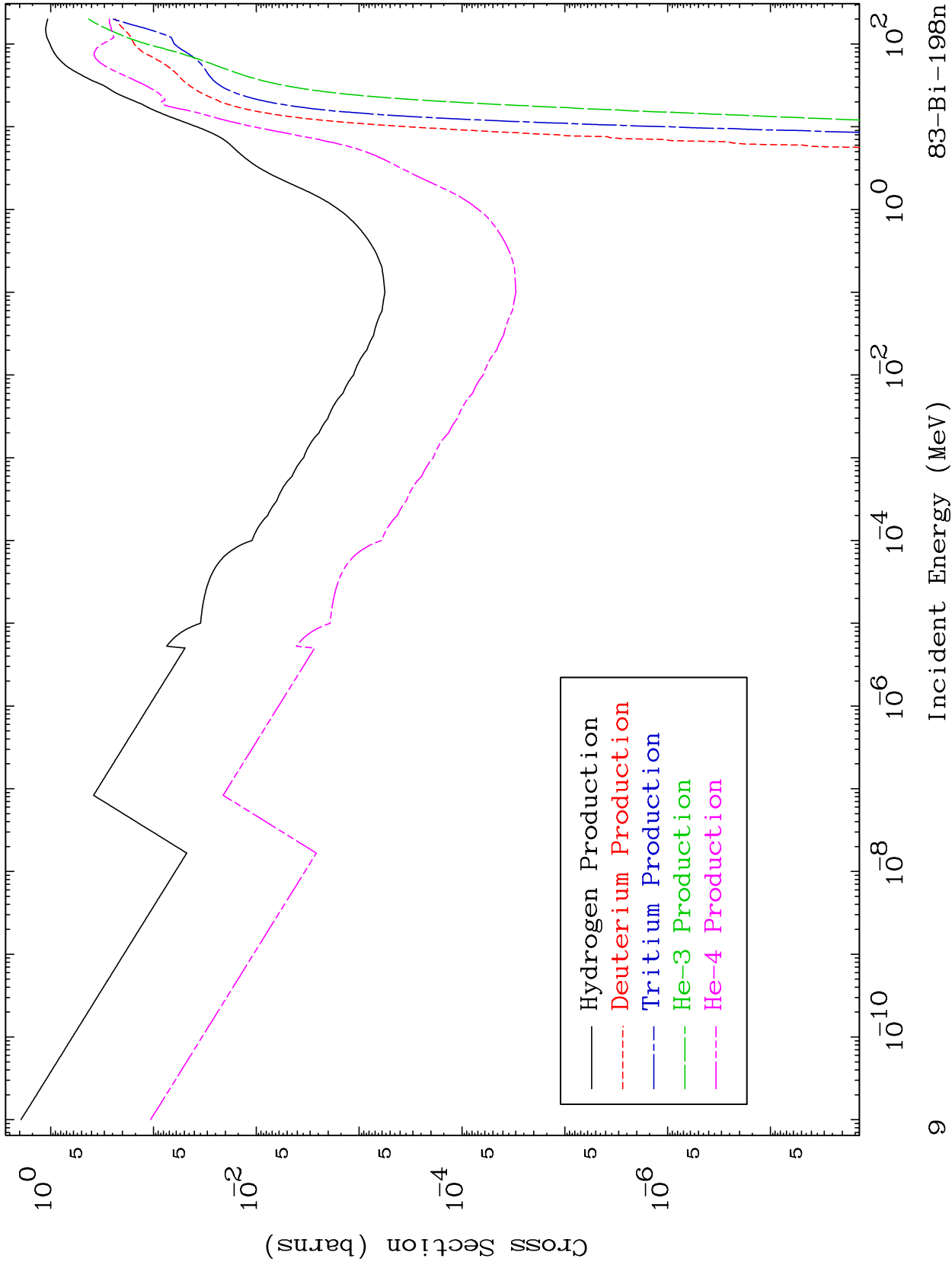
83-Bi-198n



MAT 8294

Particle Production
293 Kelvin Cross Sections

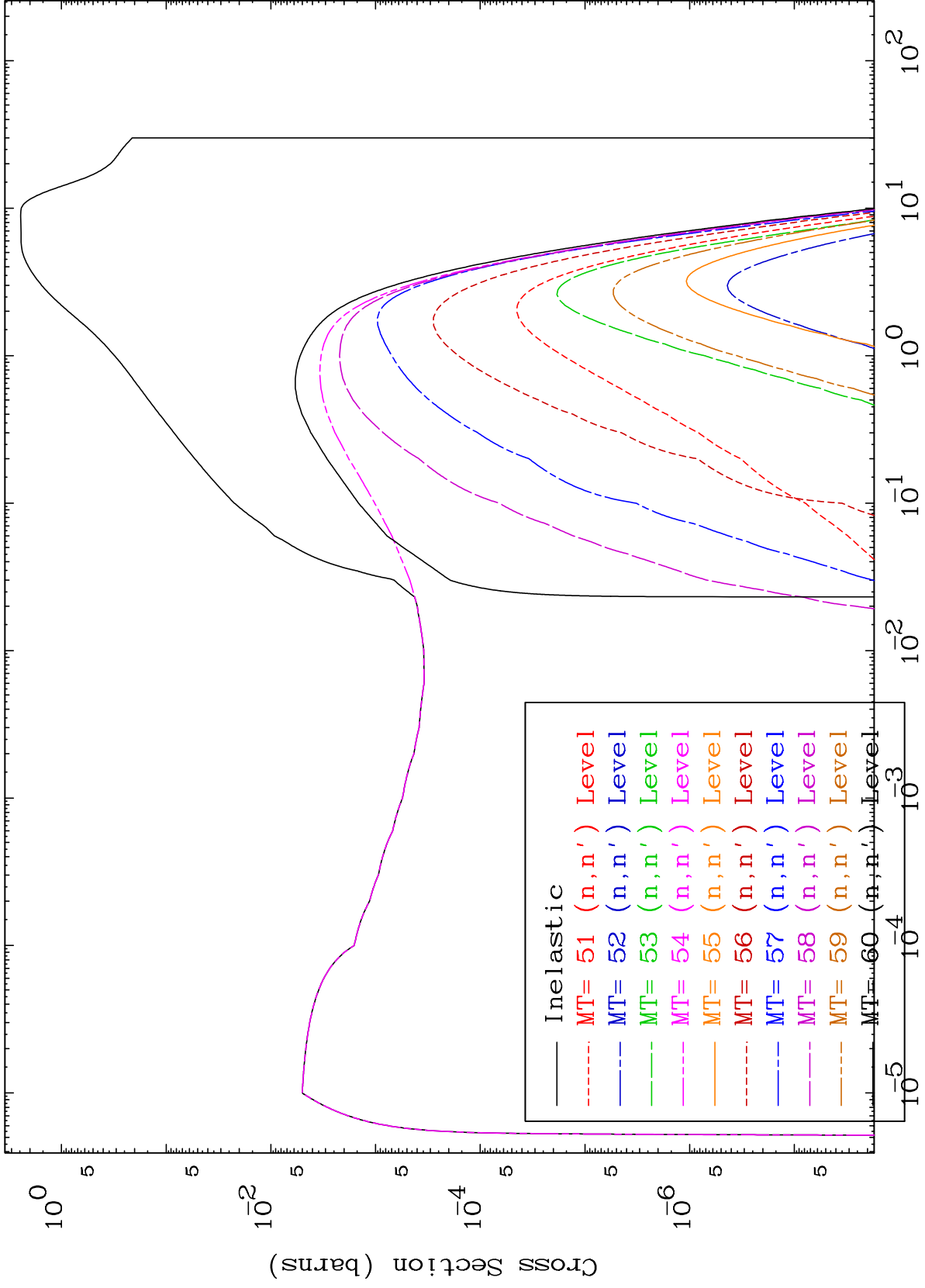
83-Bi-198n



MAT 8294

(n,n') Levels
293 Kelvin Cross Sections

83-Bi-198n



10

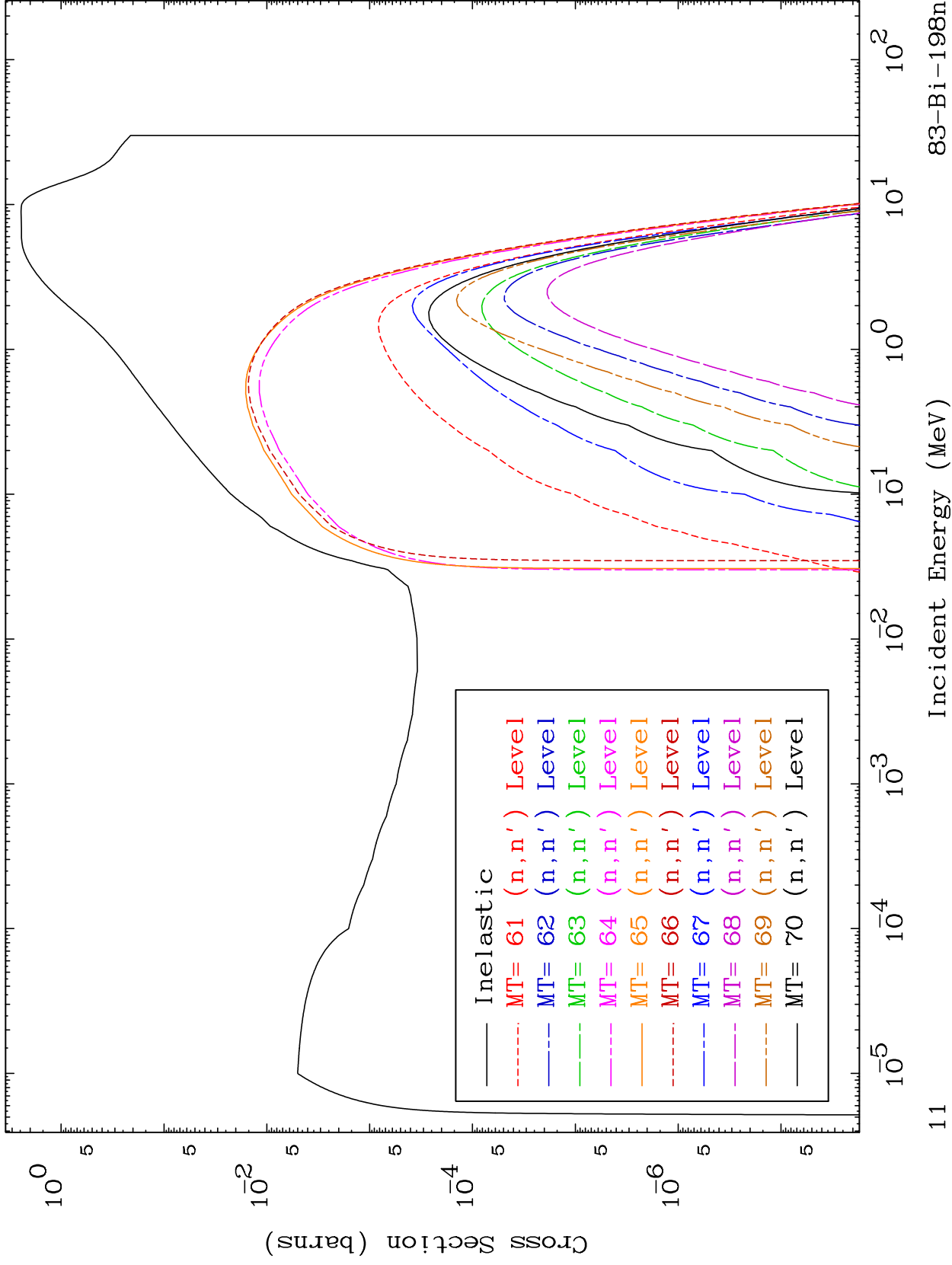
Incident Energy (MeV)

83-Bi-198n

MAT 8294

(n,n') Levels
293 Kelvin Cross Sections

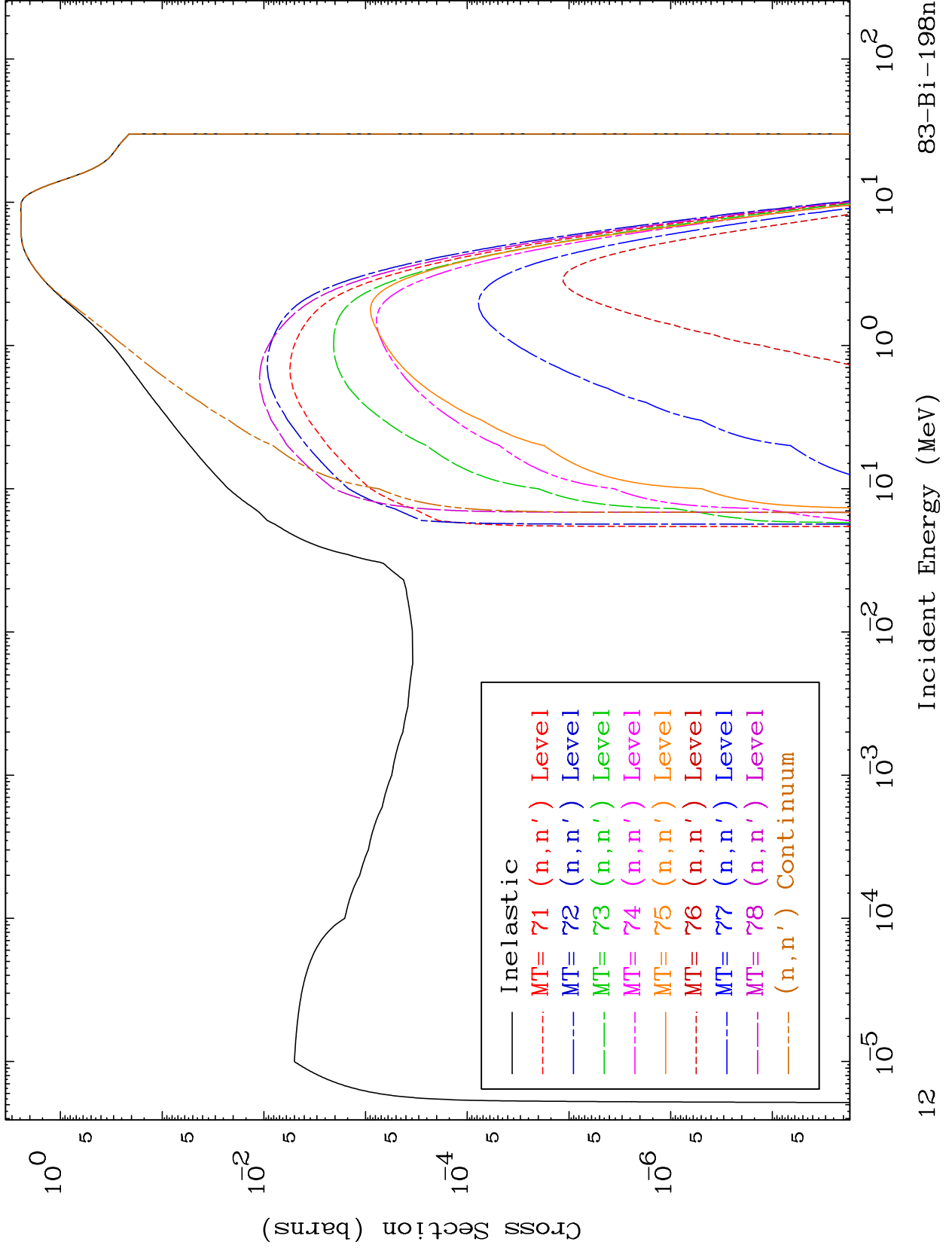
83-Bi-198n



MAT 8294

(n,n') Levels
293 Kelvin Cross Sections

83-Bi-198n



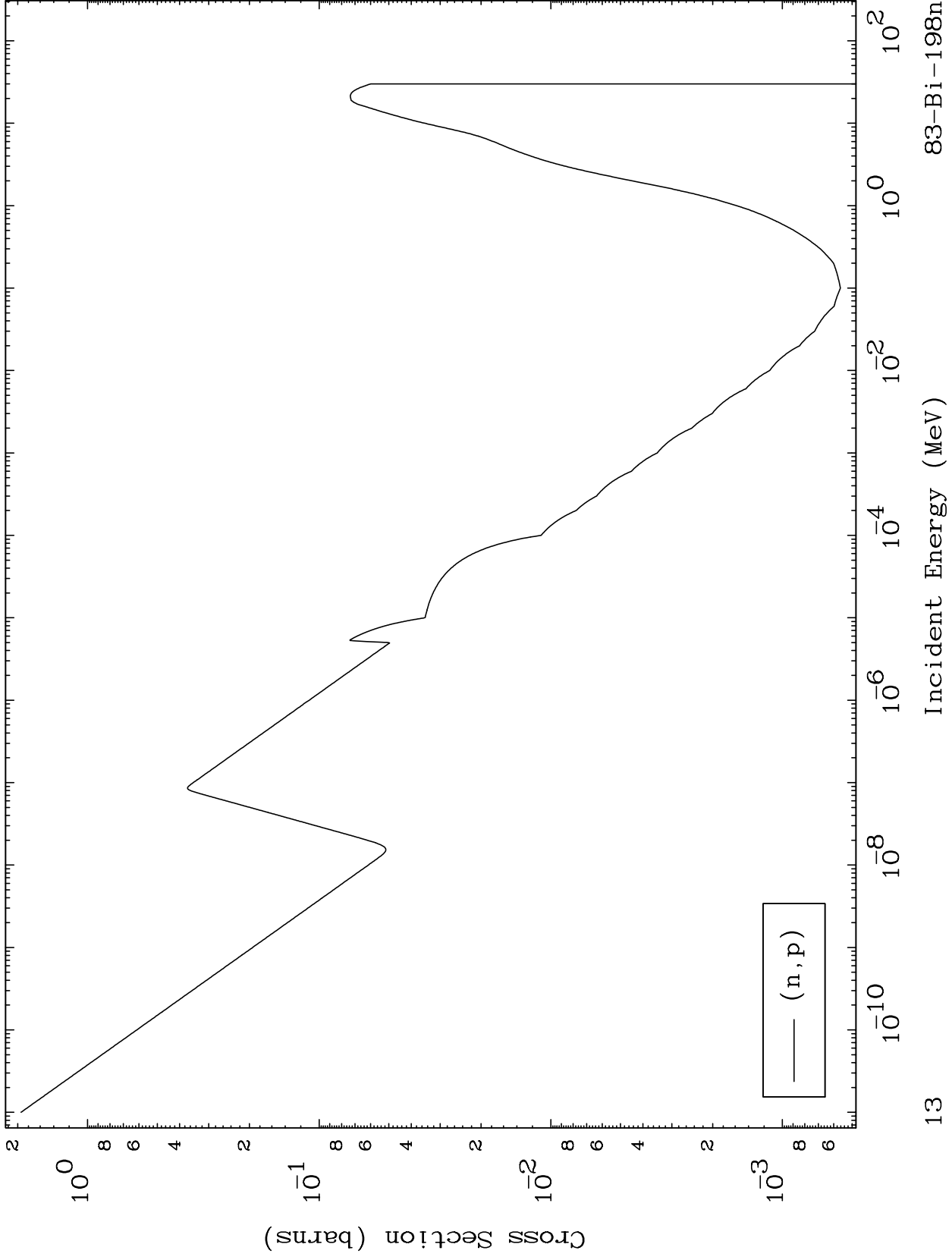
83-Bi-198n

Incident Energy (MeV)

MAT 8294

(n,p) Levels
293 Kelvin Cross Sections

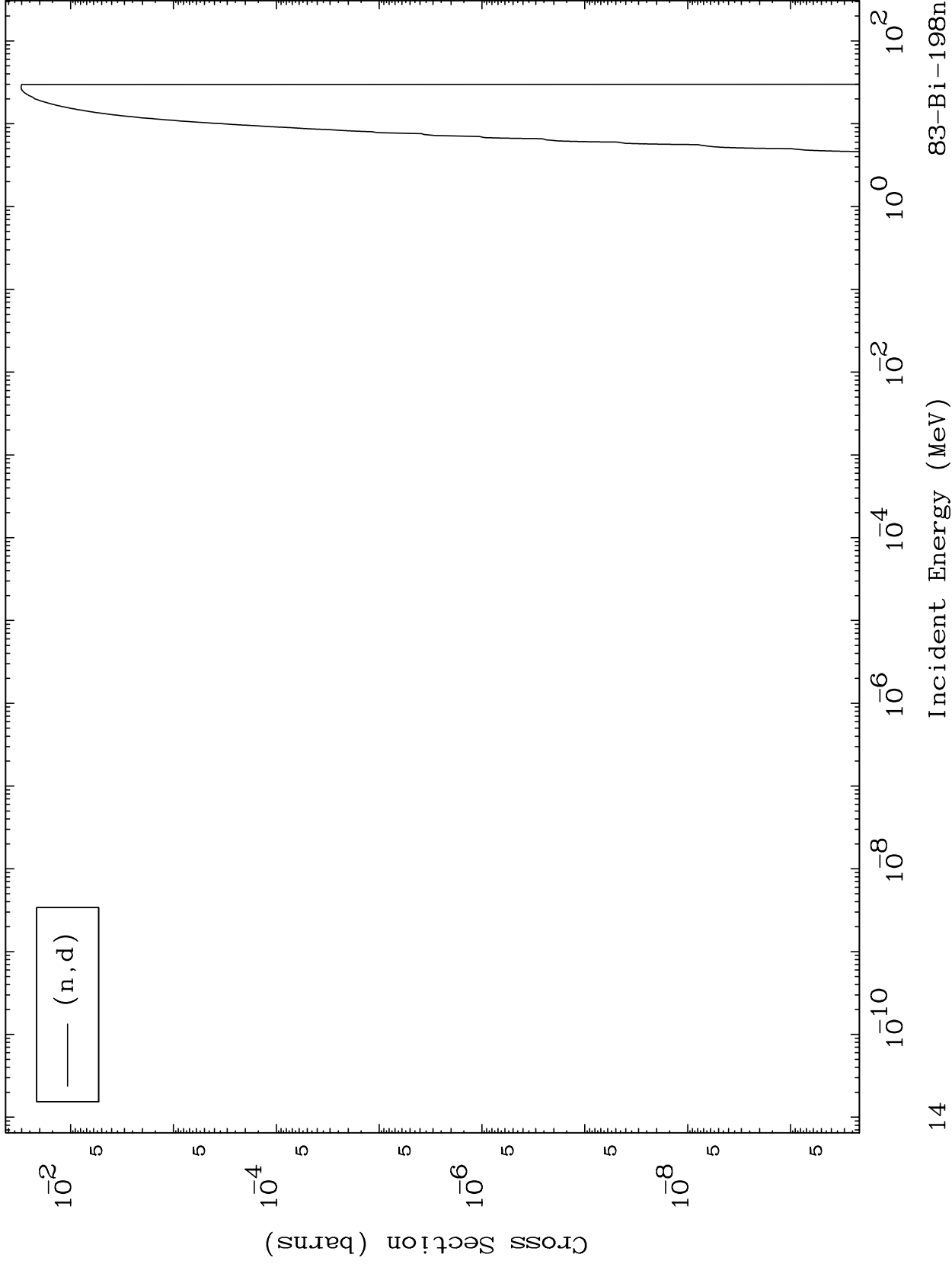
83-Bi-198n



MAT 8294

(n,d) Levels
293 Kelvin Cross Sections

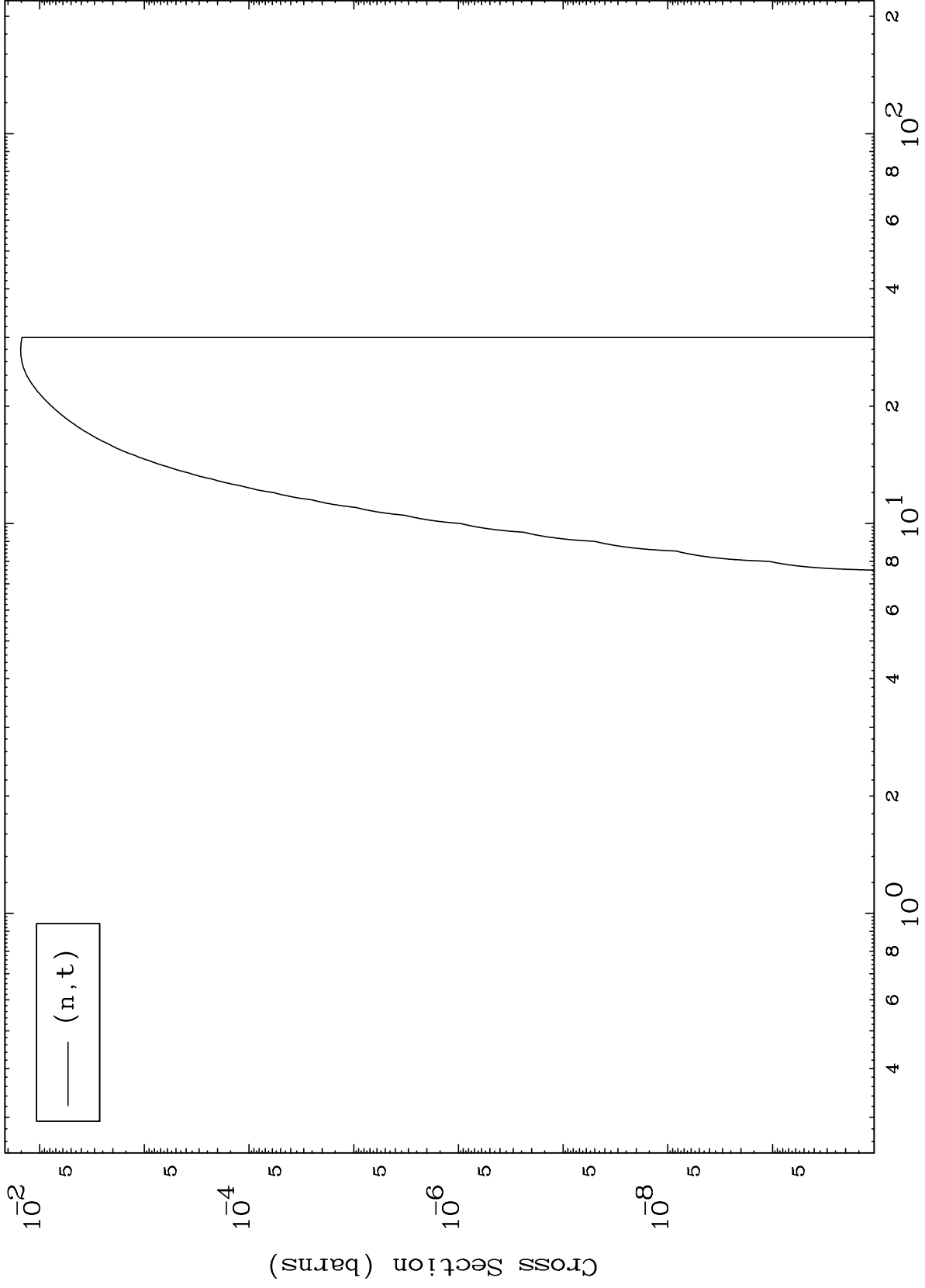
83-Bi-198n



MAT 8294

(n,t) Levels
293 Kelvin Cross Sections

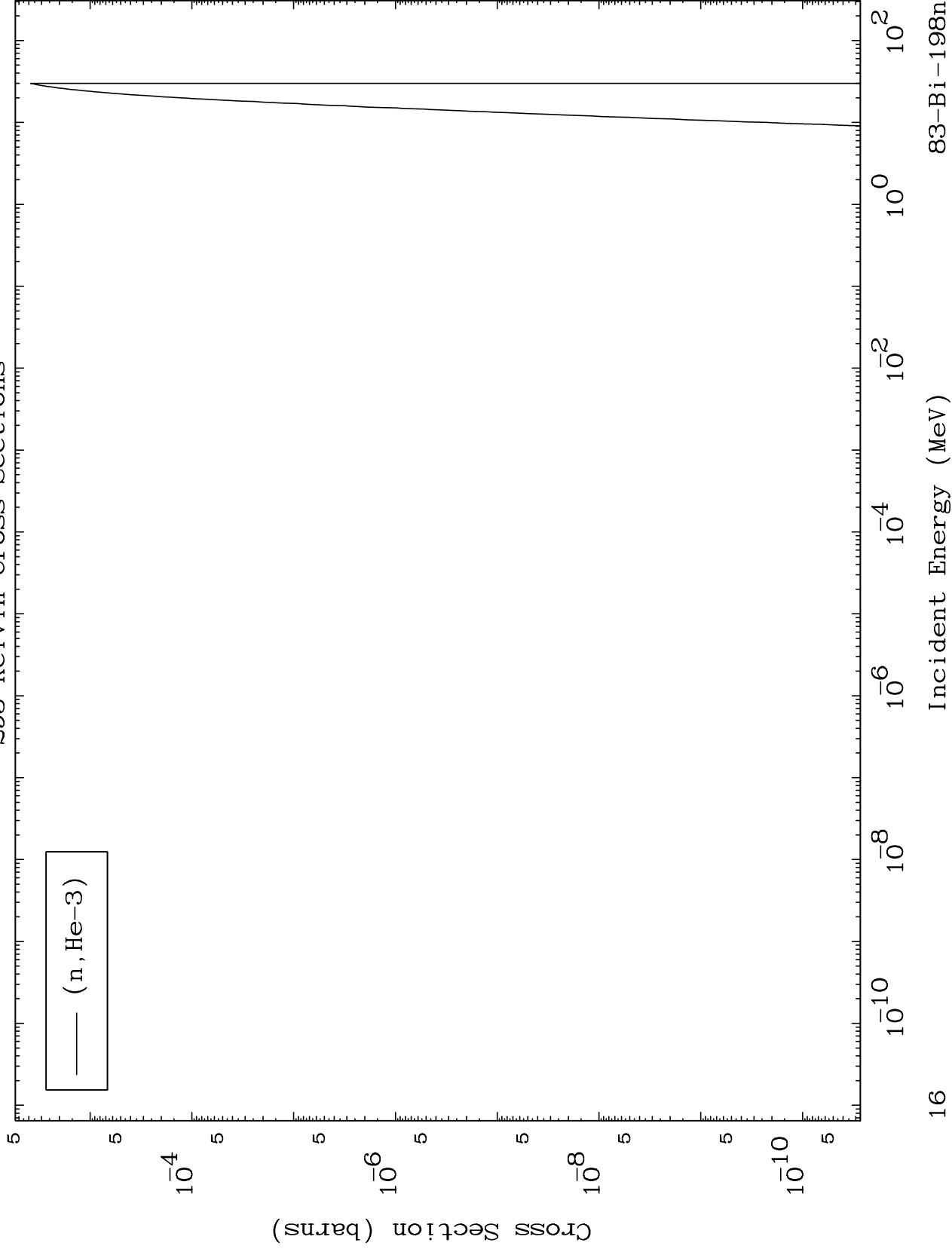
83-Bi-198n



MAT 8294

(n,He3) Levels
293 Kelvin Cross Sections

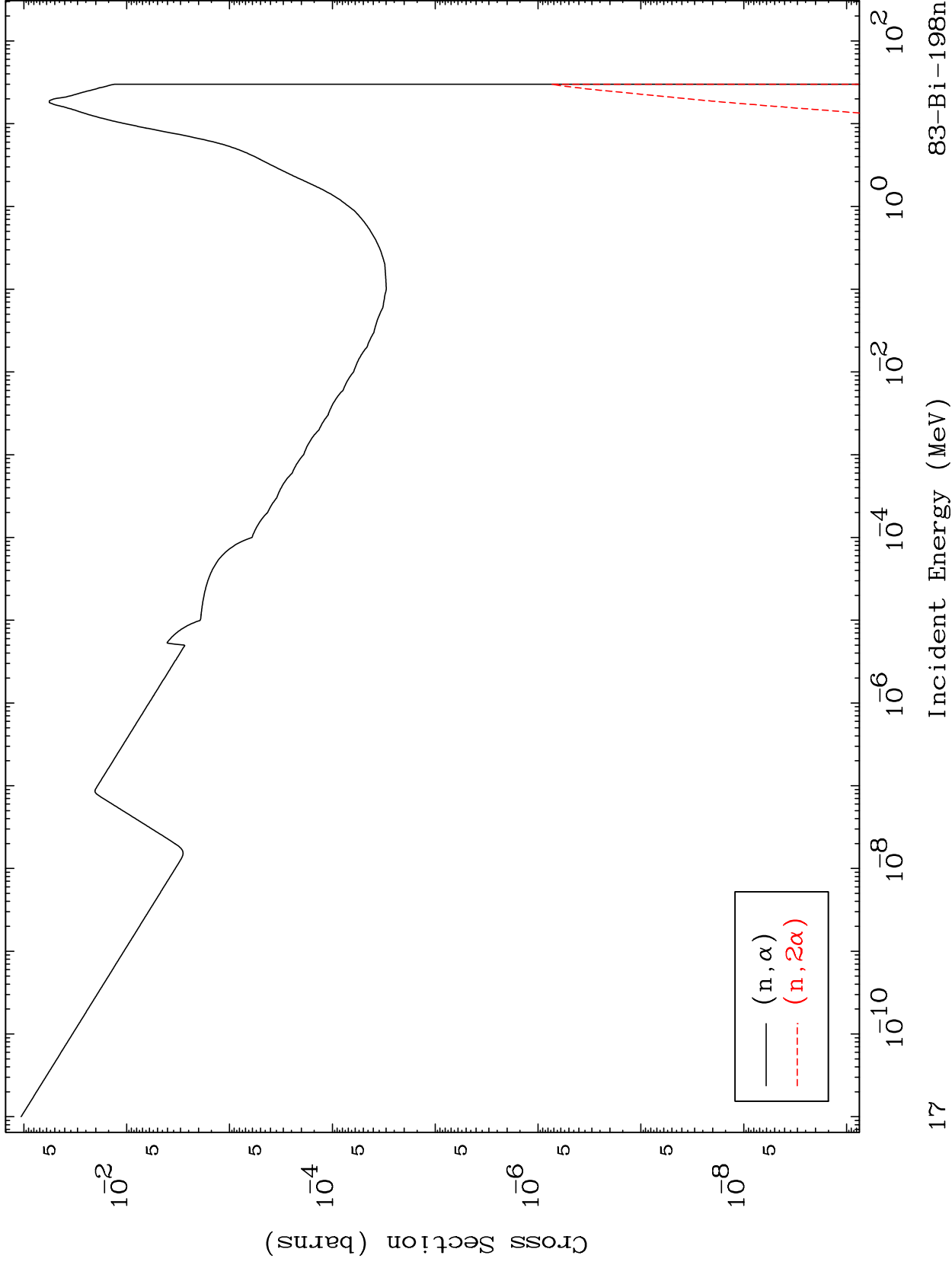
83-Bi-198n



MAT 8294

(n,α) Levels
293 Kelvin Cross Sections

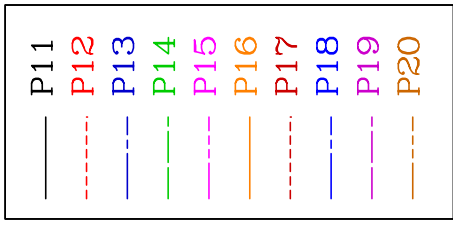
83-Bi-198n



MAT 8294

Elastic Legendre Coefficients

83-Bi-198n



$\times 10^{-4}$
2.0
1.5
1.0
0.5
0.0

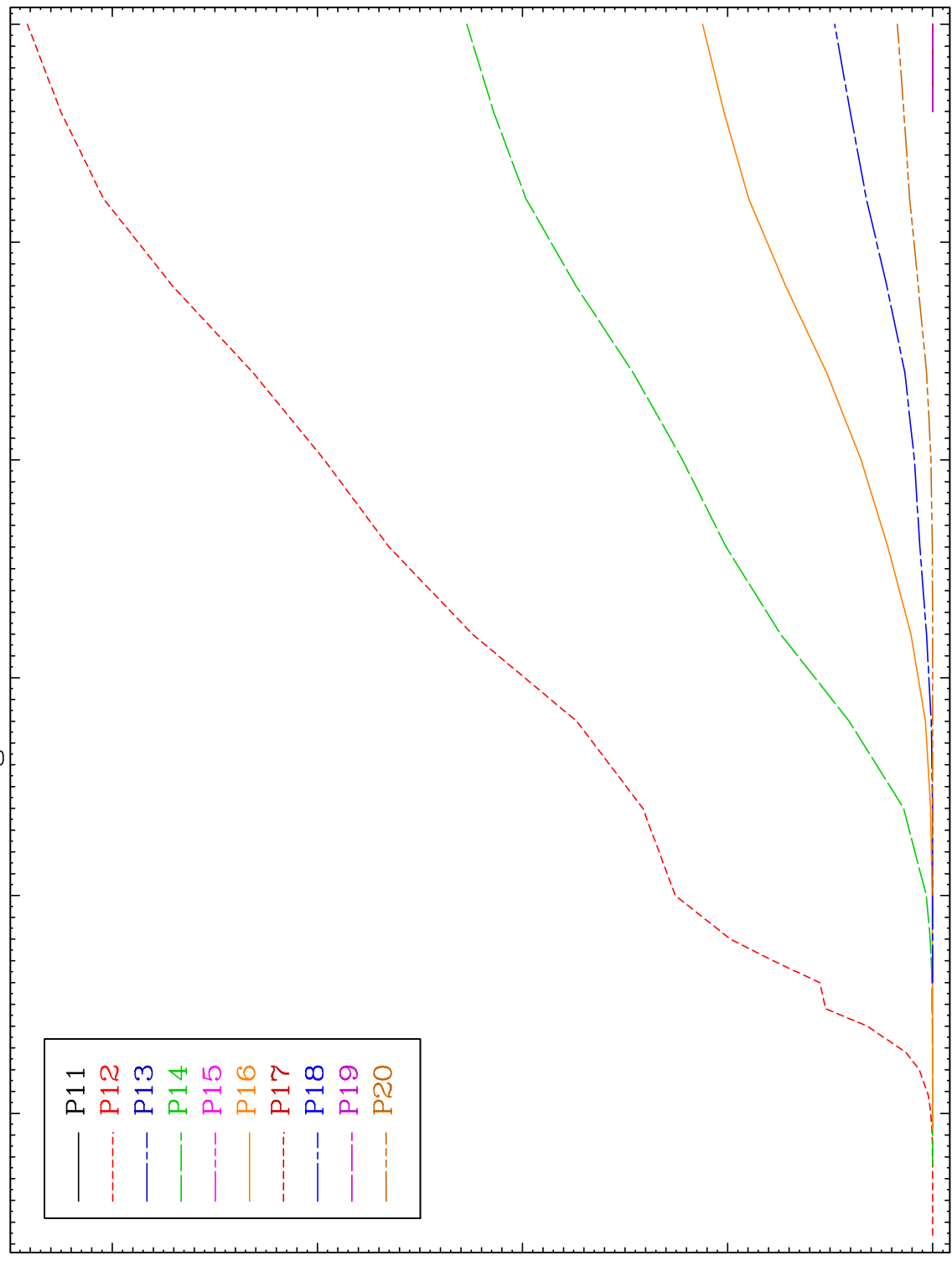
Legendre (CM)

5 10 15 20 25 30

19

Incident Energy (MeV)

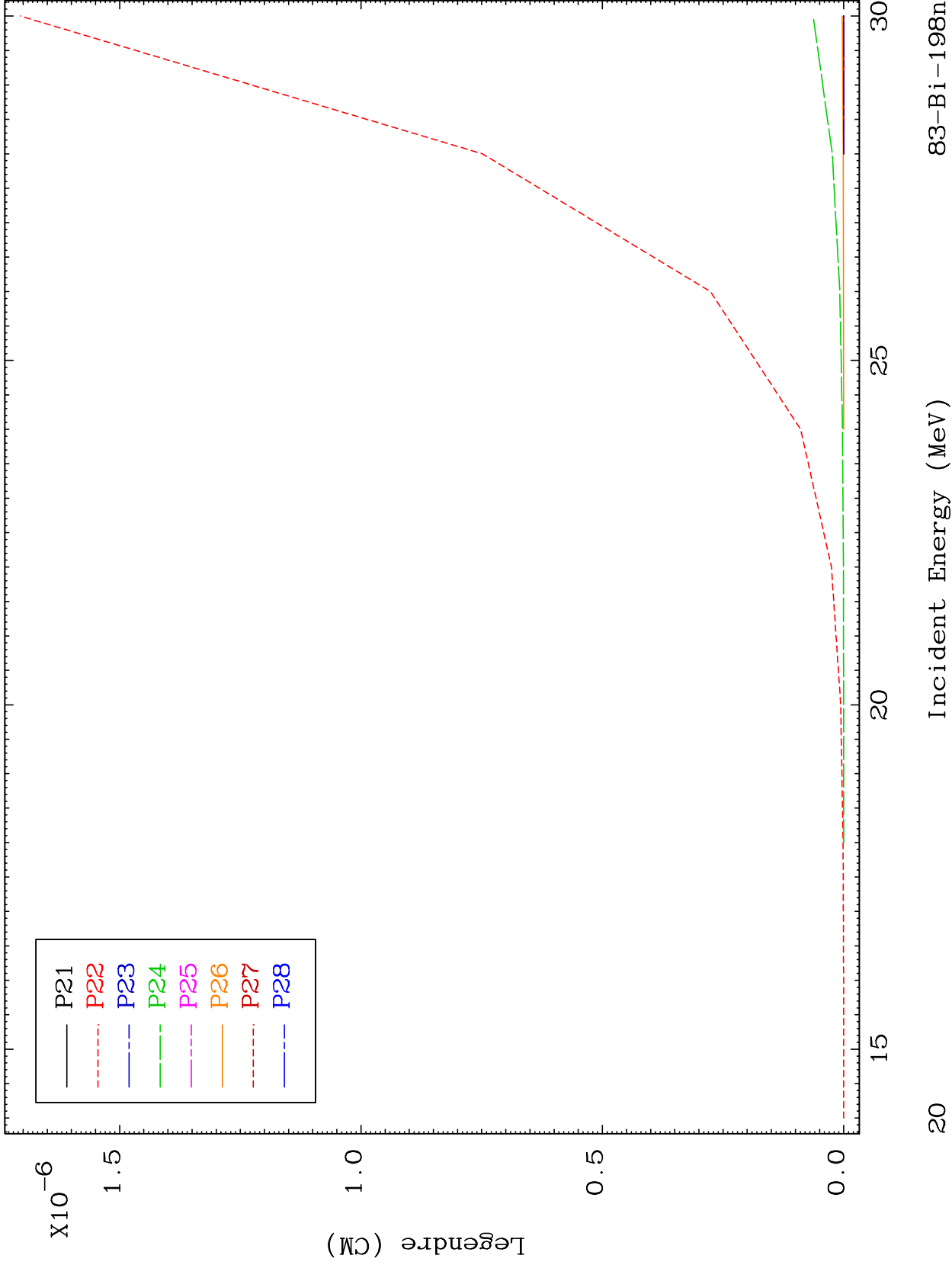
83-Bi-198n



MAT 8294

Elastic
Legendre Coefficients

83-Bi-198n

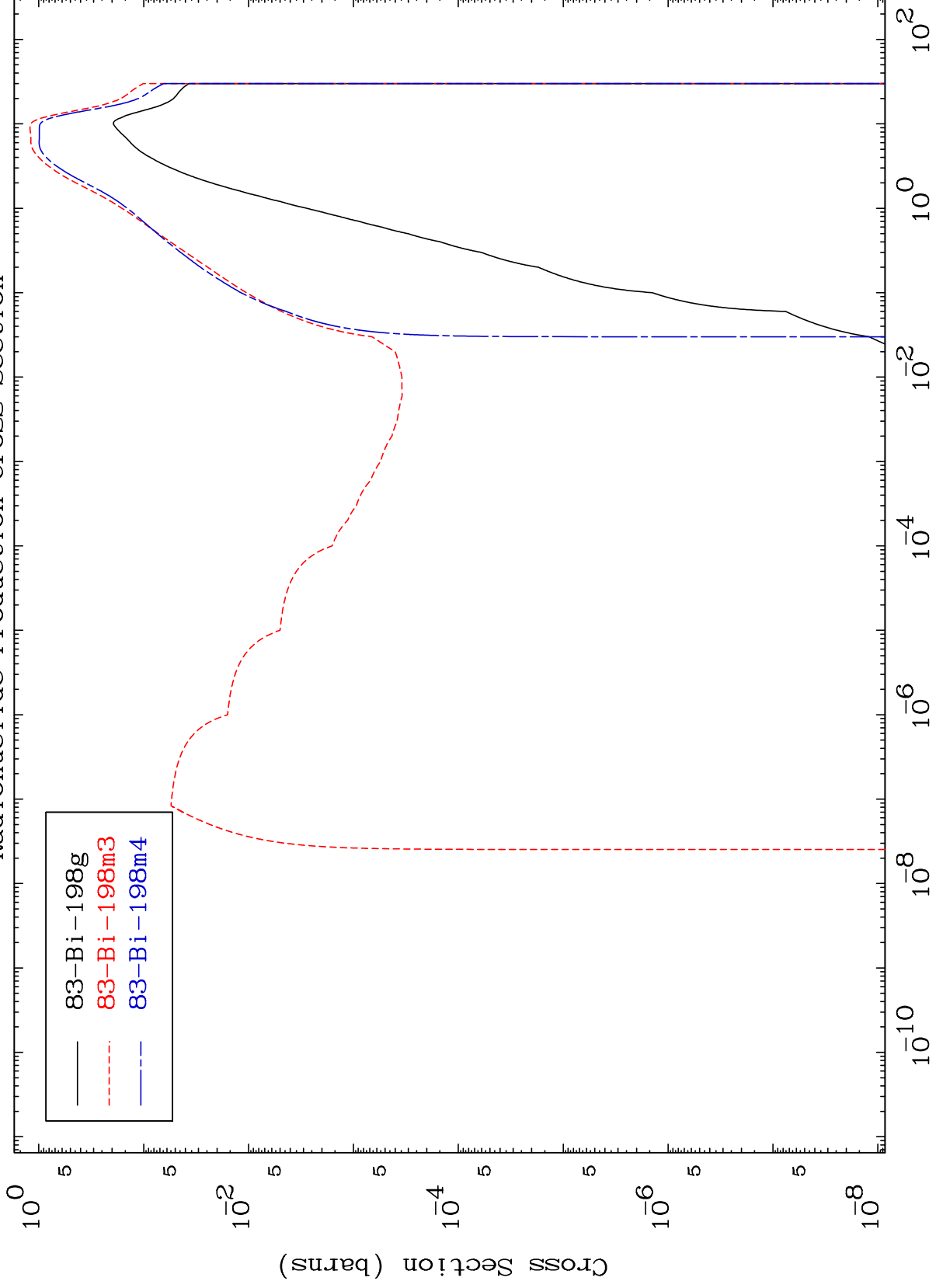


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83-Bi-198n

Inelastic

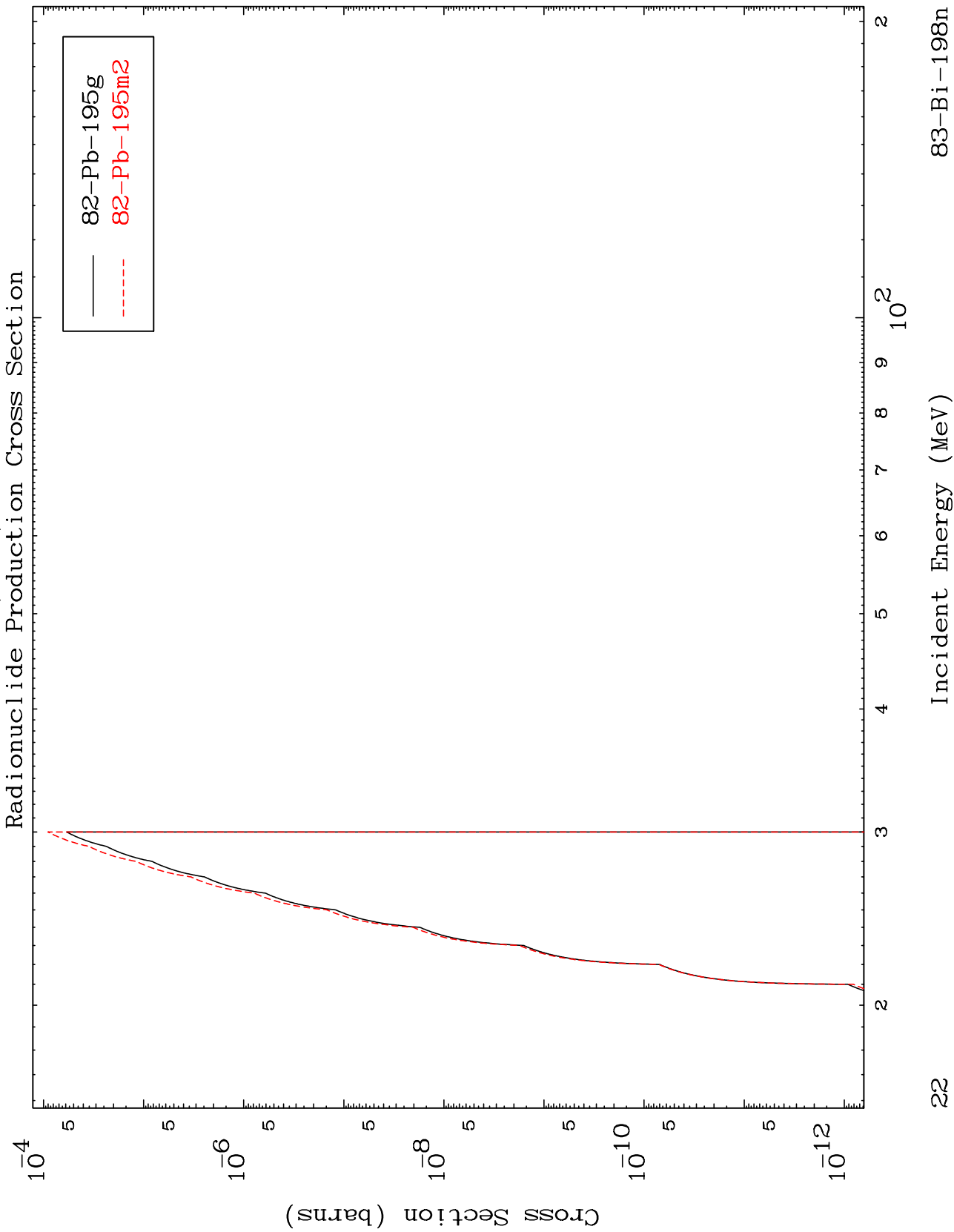
Radionuclide Production Cross Section



MAT 8294

(n,2n) d

83-Bi-198n



22

Incident Energy (MeV)

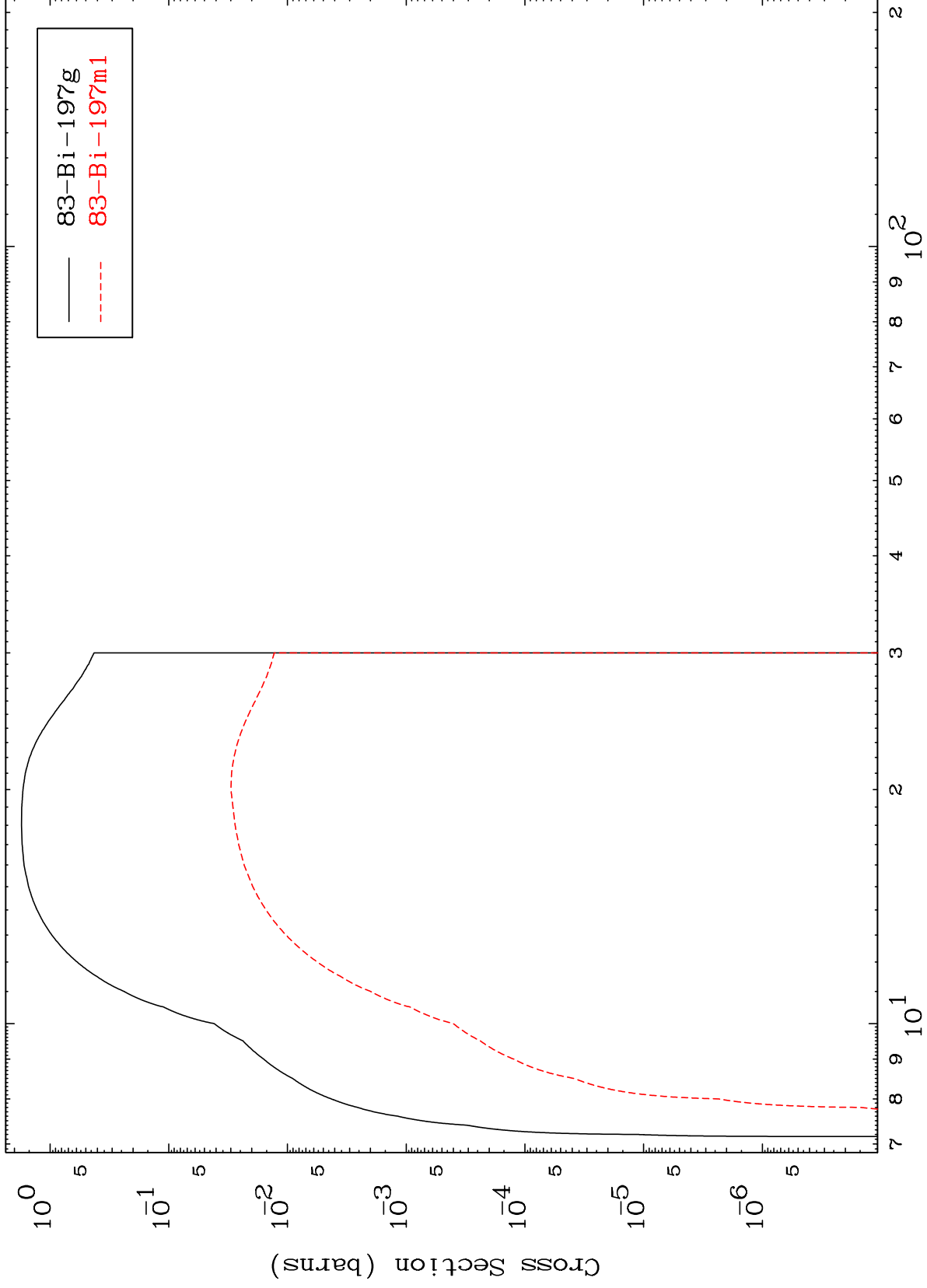
83-Bi-198n

MAT 8294

83-Bi-198n

(n,2n)

Radionuclide Production Cross Section



23

Incident Energy (MeV)

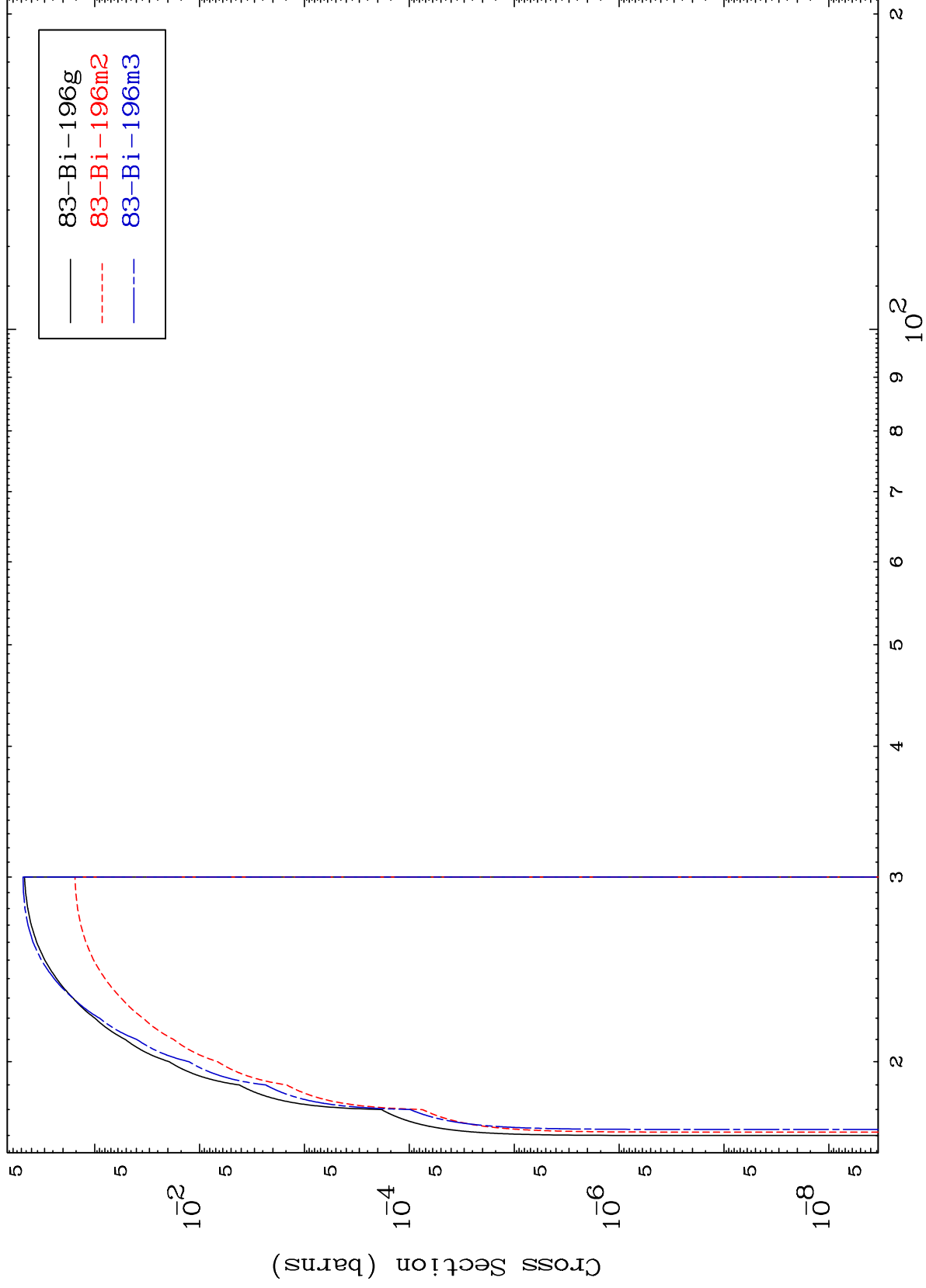
83-Bi-198n

MAT 8294

(n,3n)

⁸³Bi-198n

Radionuclide Production Cross Section



24

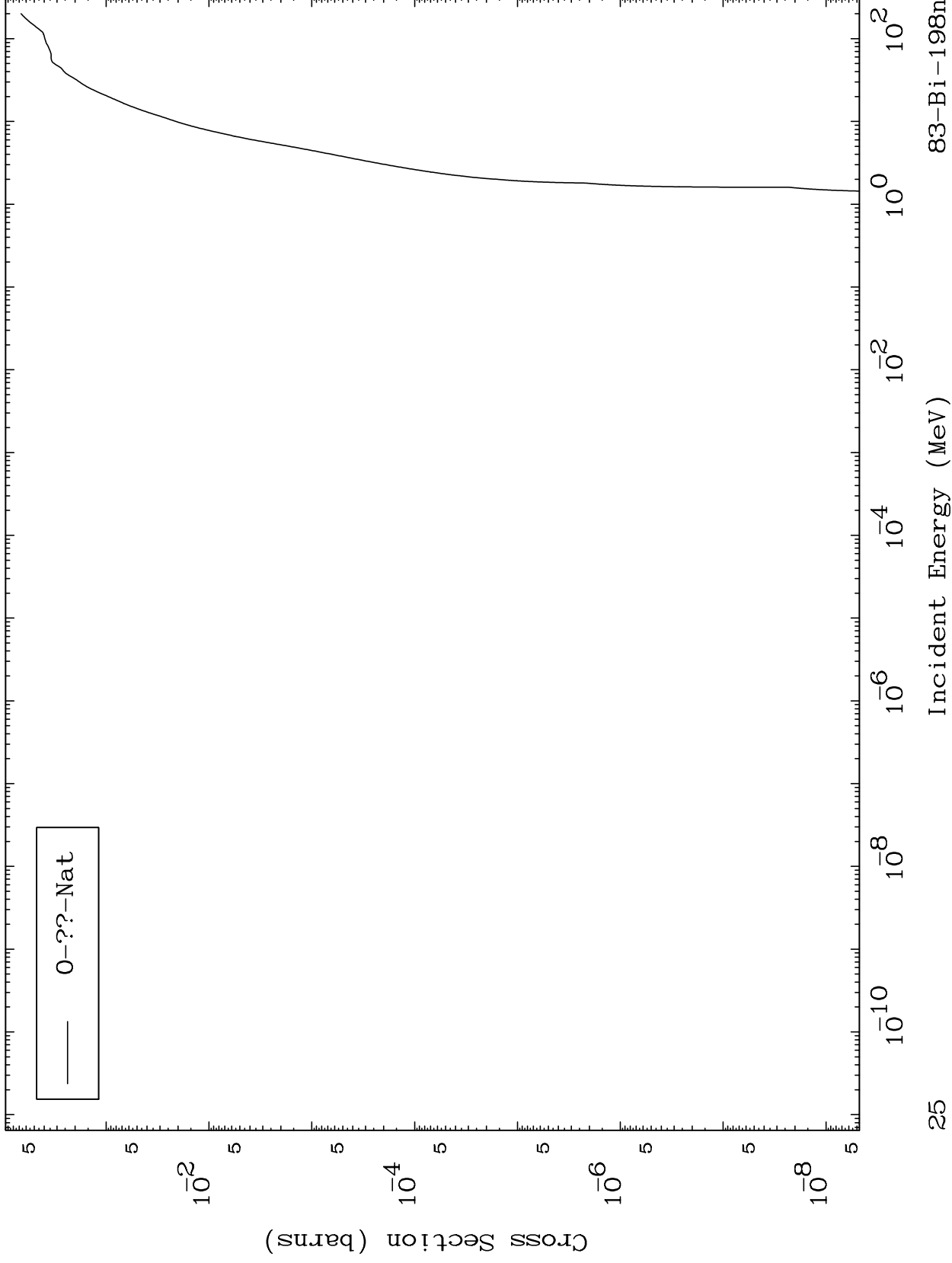
Incident Energy (MeV)

⁸³Bi-198n

MAT 8294

Fission
Radionuclide Production Cross Section

83-Bi-198n



25

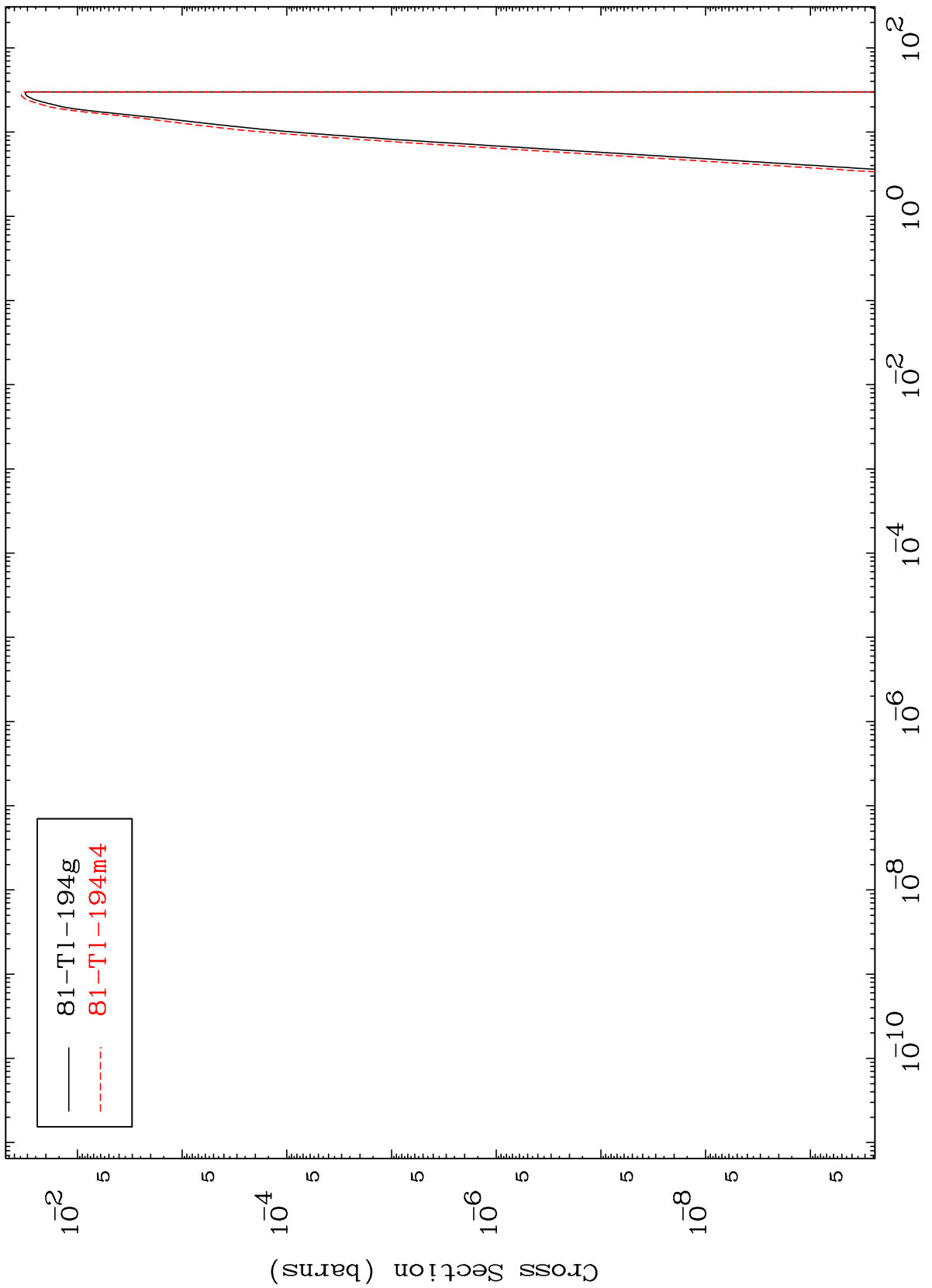
83-Bi-198n

MAT 8294

(n,n') α

83-Bi-198n

Radionuclide Production Cross Section

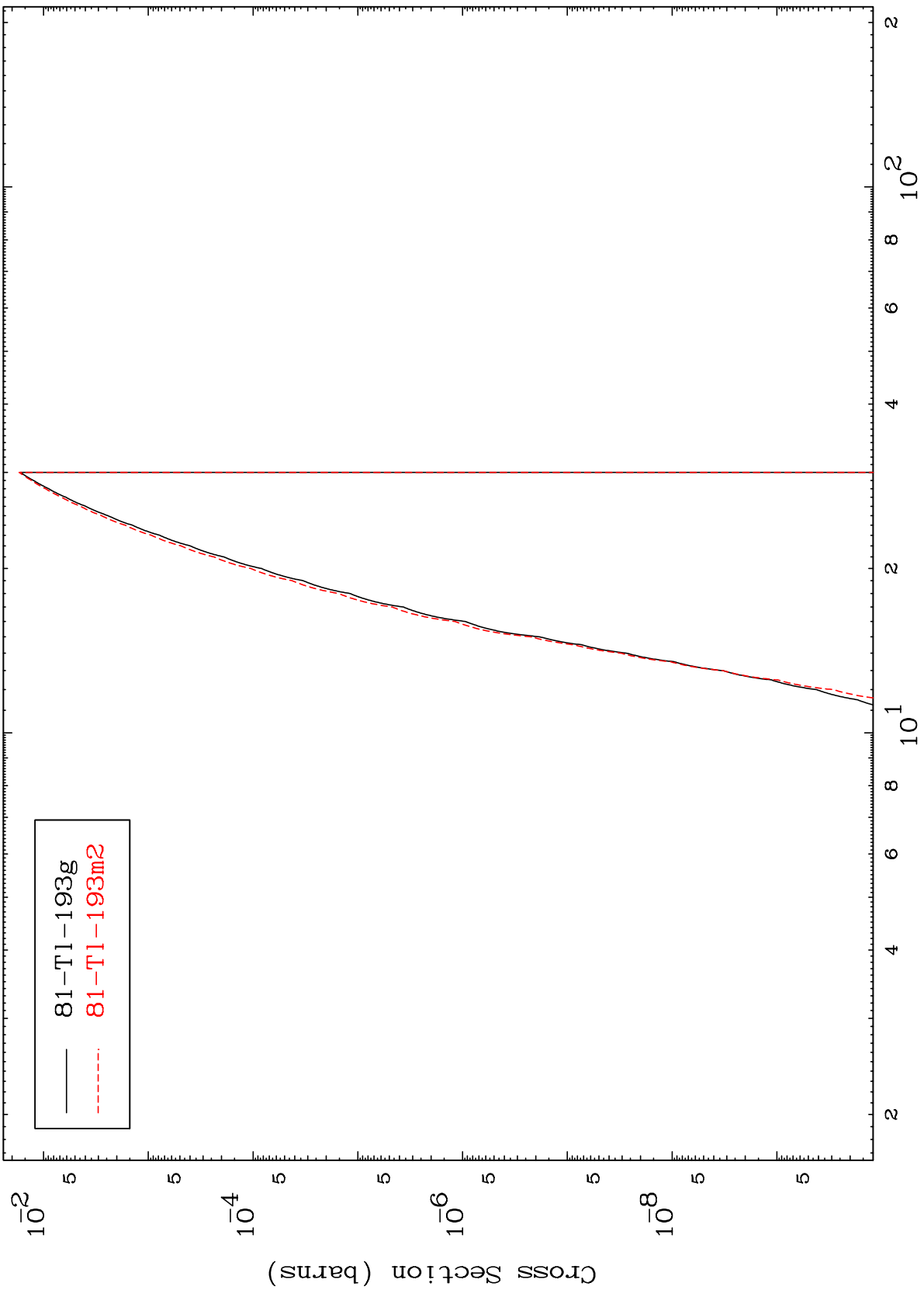


MAT 8294

(n,2n) α

83-Bi-198n

Radionuclide Production Cross Section



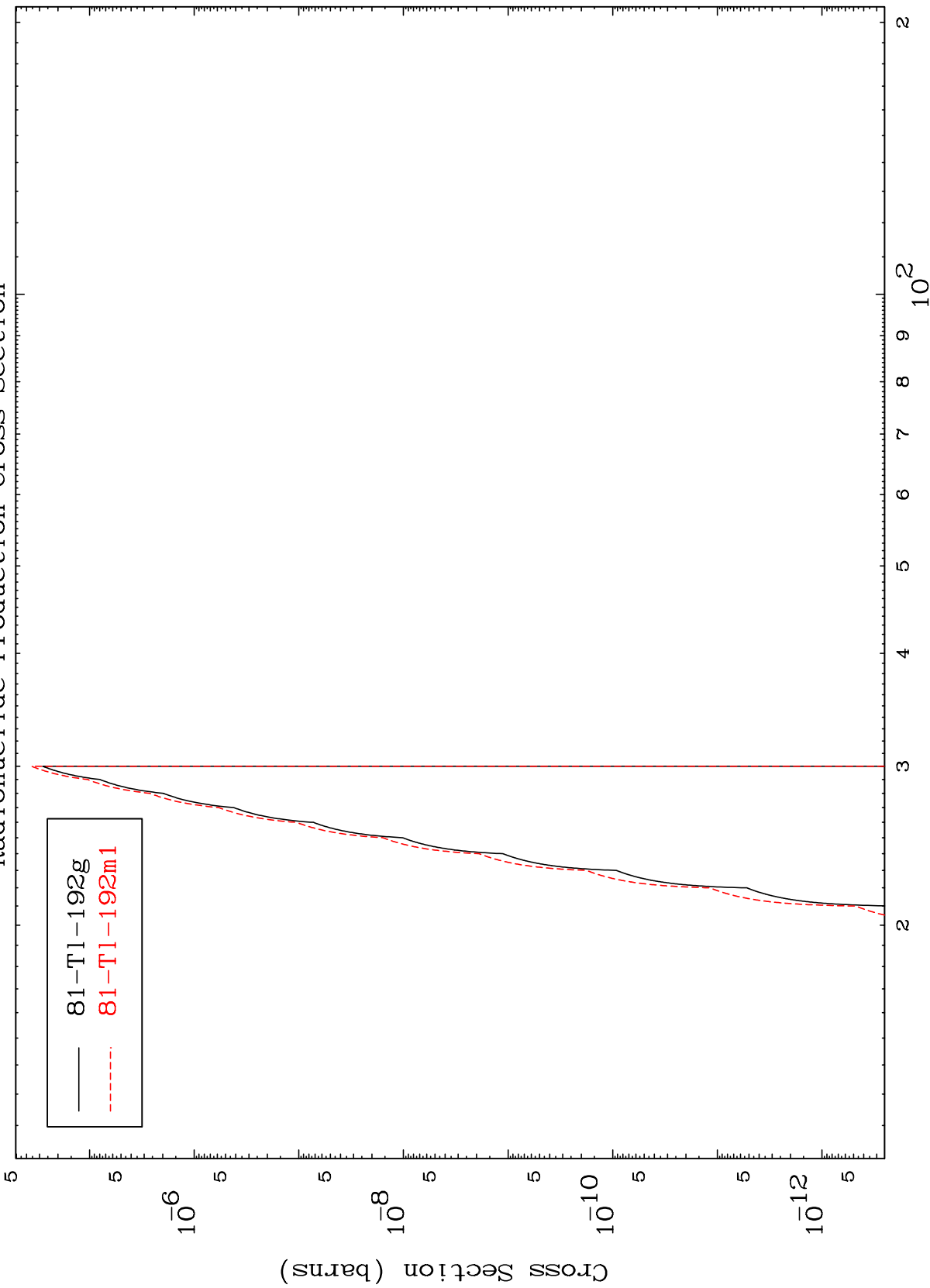
81-Tl-193g
81-Tl-193m2

MAT 8294

83-Bi-198n

(n,3n) α

Radionuclide Production Cross Section

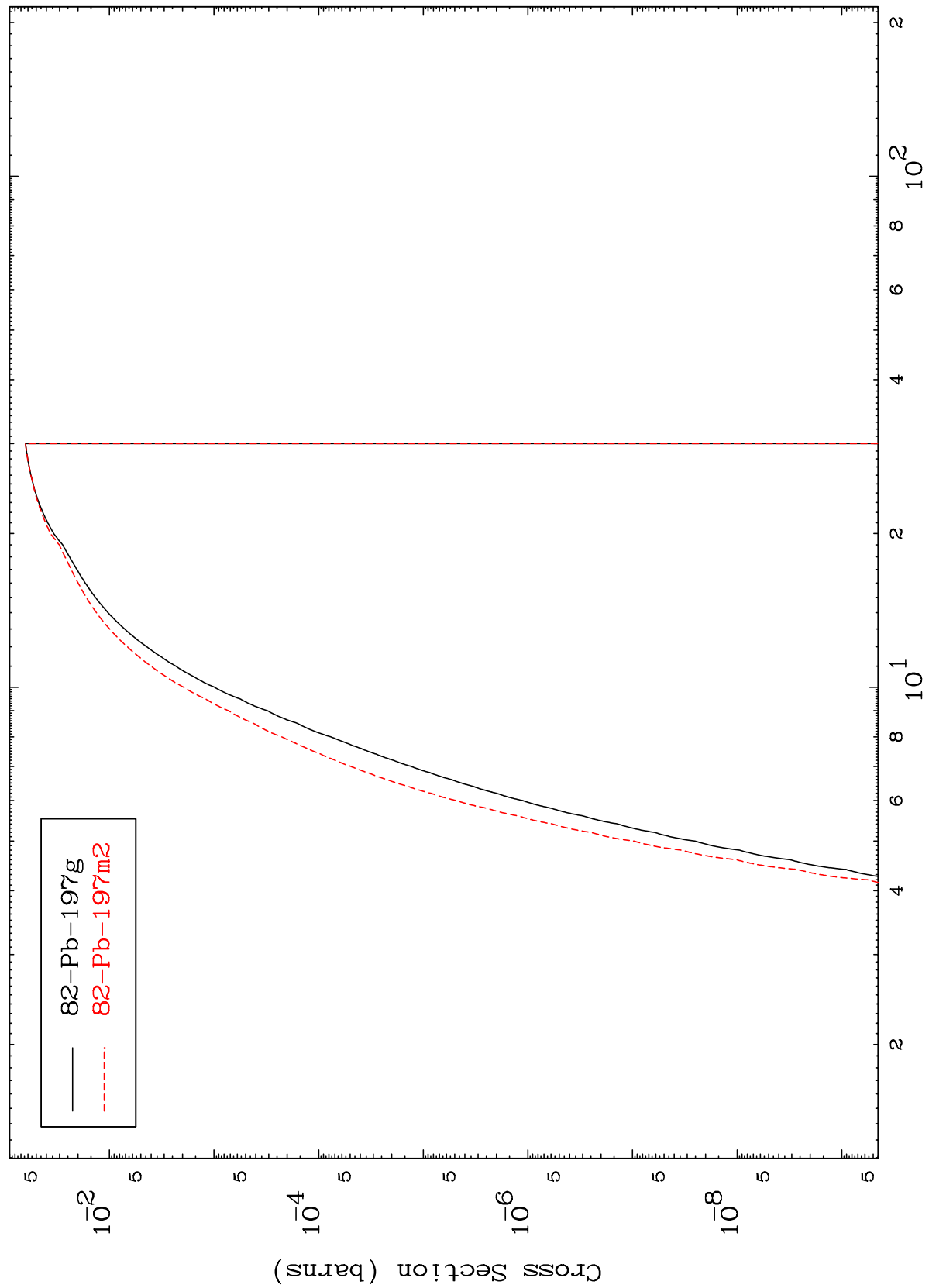


MAT 8294

(n,n') p

83-Bi-198n

Radionuclide Production Cross Section



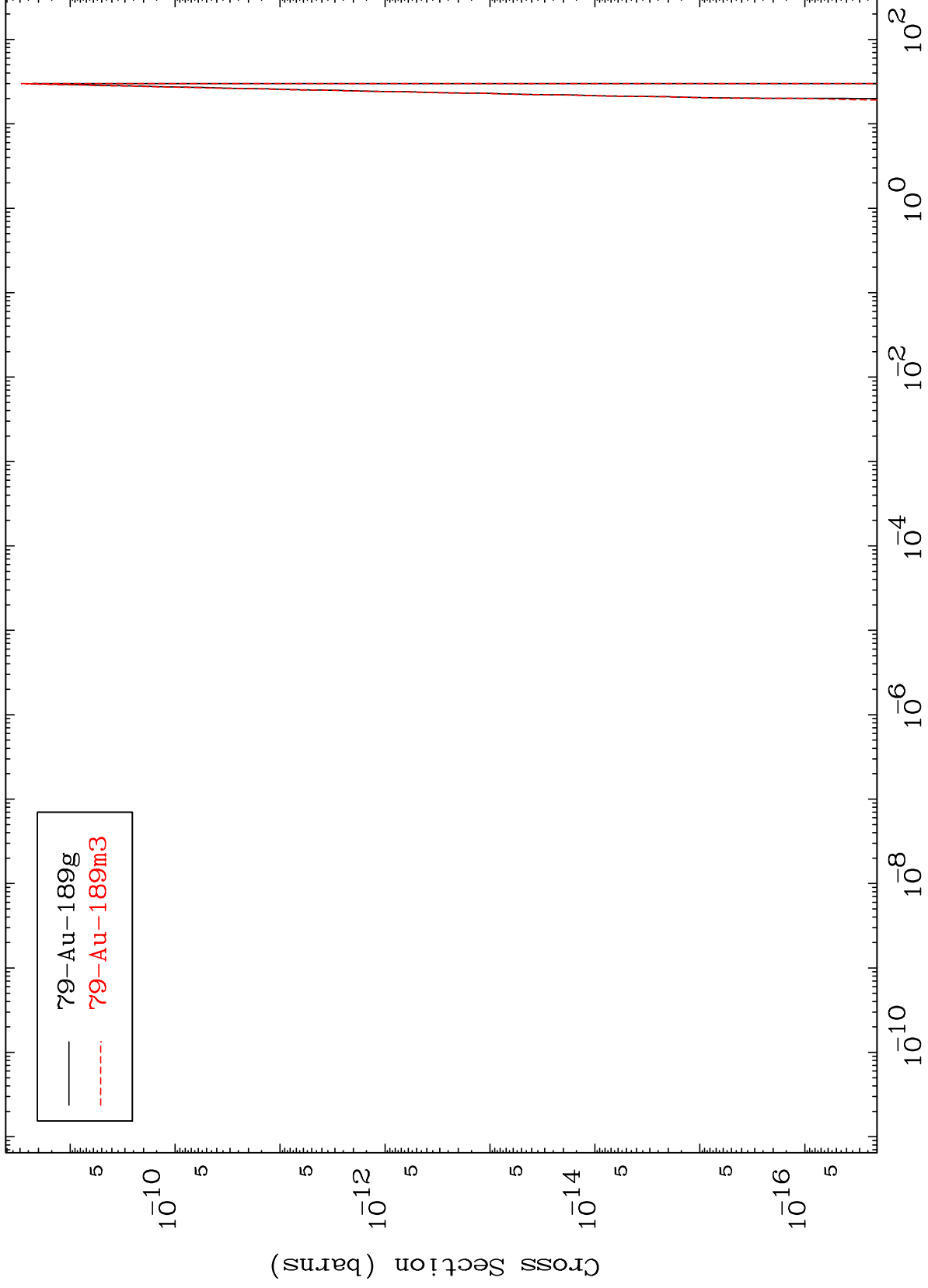
82-Pb-197g
82-Pb-197m2

MAT 8294

(n,2n) 2 α

83-Bi-198n

Radionuclide Production Cross Section



30

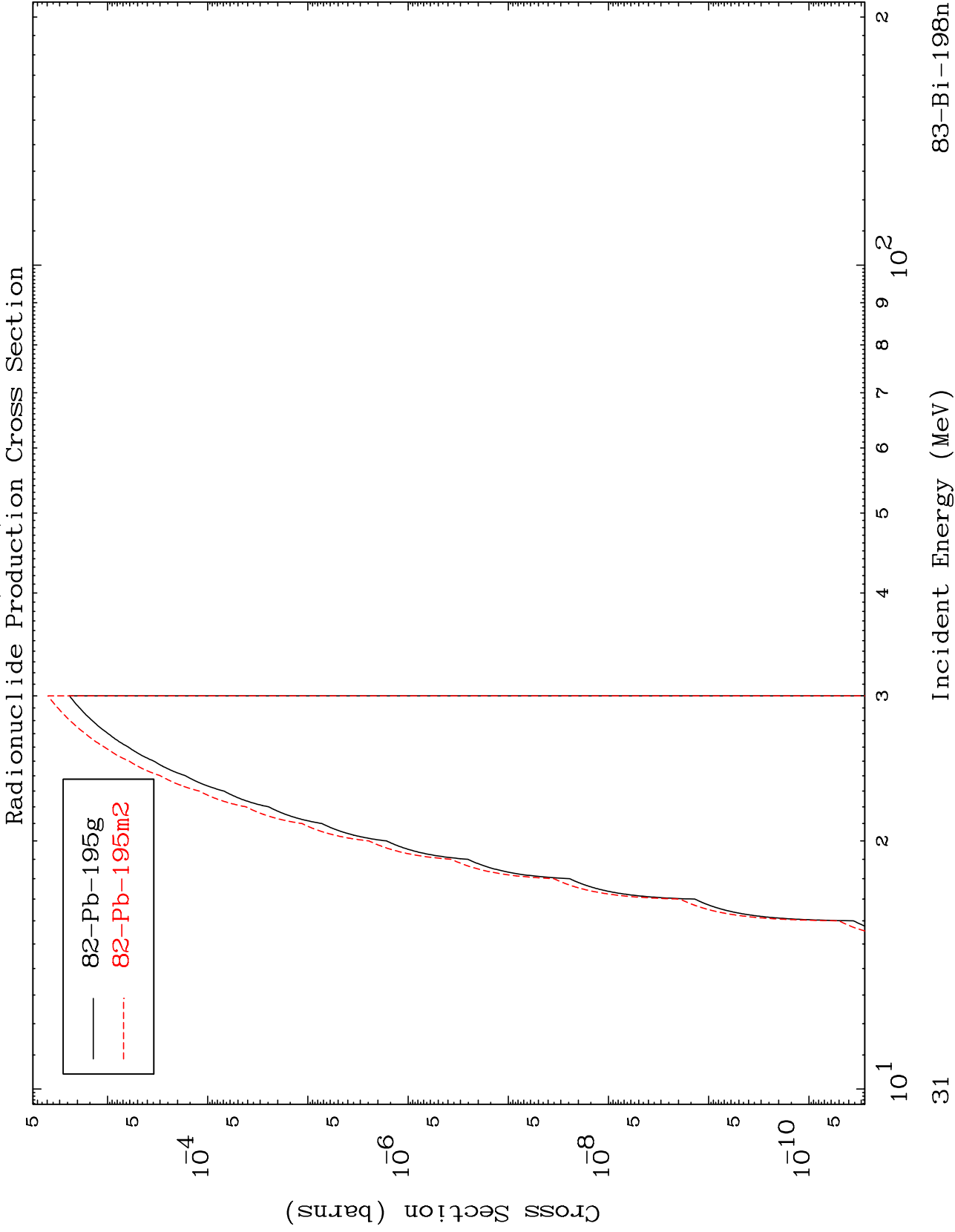
Incident Energy (MeV)

83-Bi-198n

MAT 8294

(n,n') t

83-Bi-198n



31

Incident Energy (MeV)

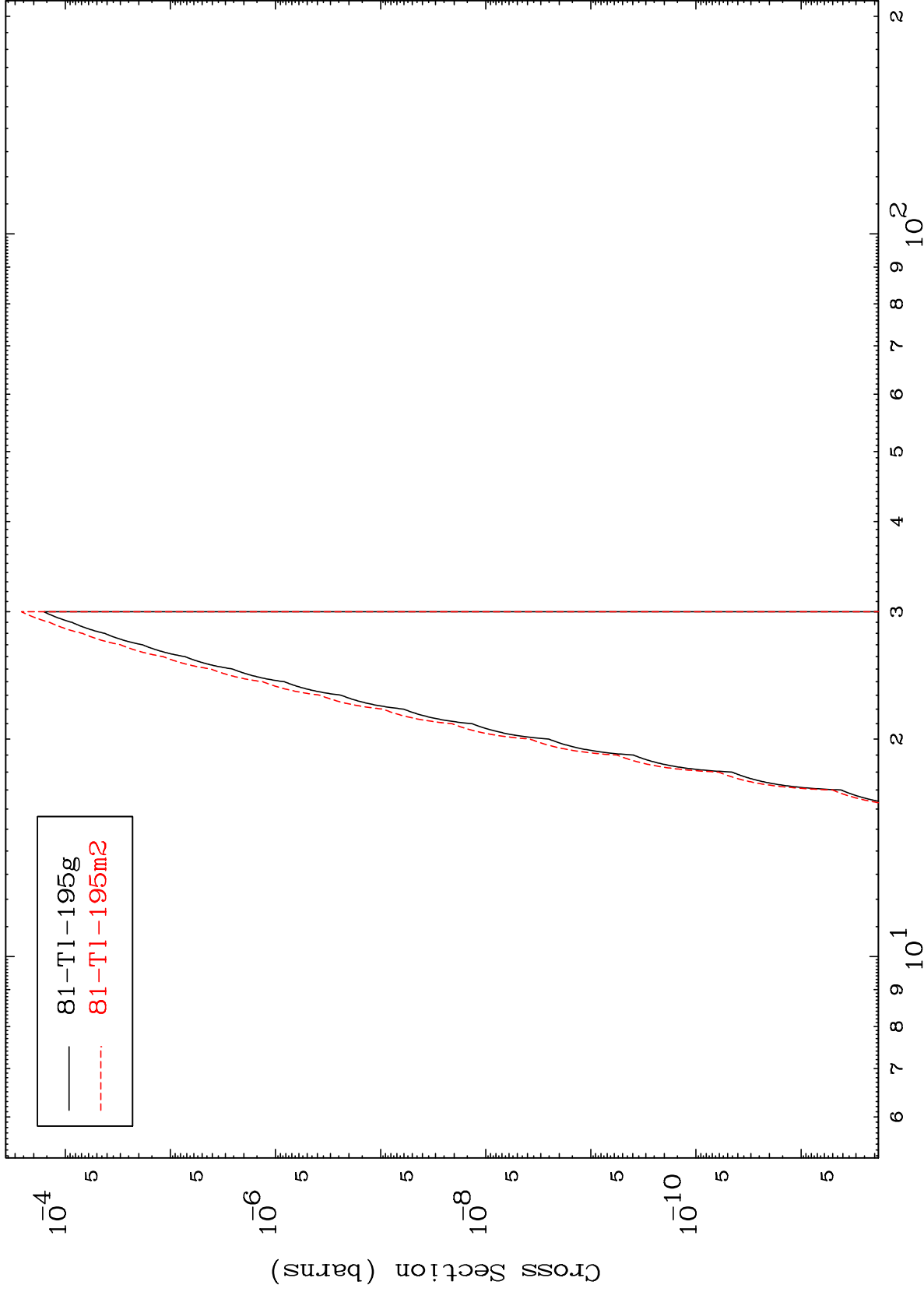
83-Bi-198n

MAT 8294

(n,n') He-3

83-Bi-198n

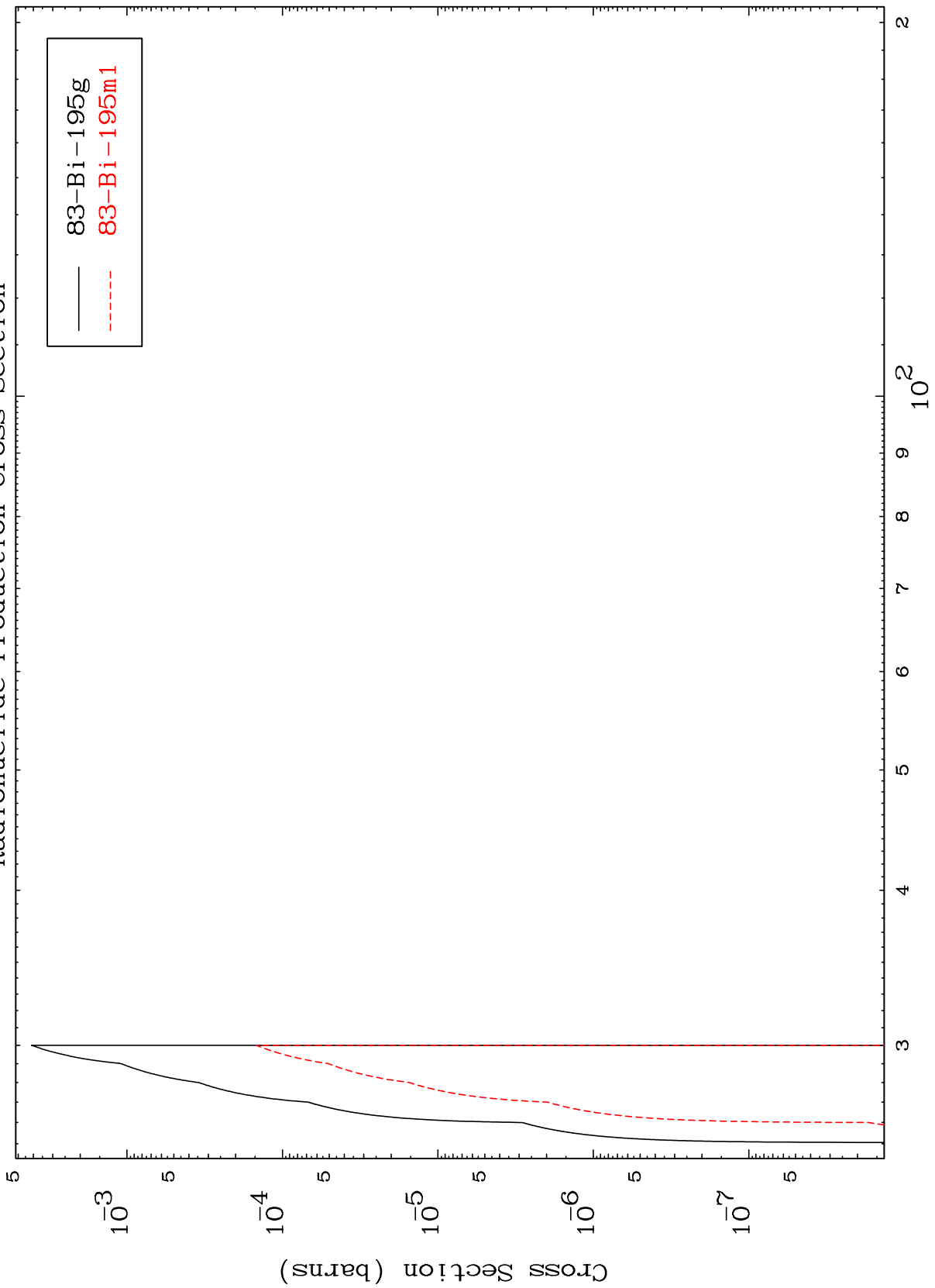
Radionuclide Production Cross Section



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$^{83}\text{Bi}-198\text{n}$

$(n,4n)$
Radionuclide Production Cross Section



33

$^{83}\text{Bi}-198\text{n}$

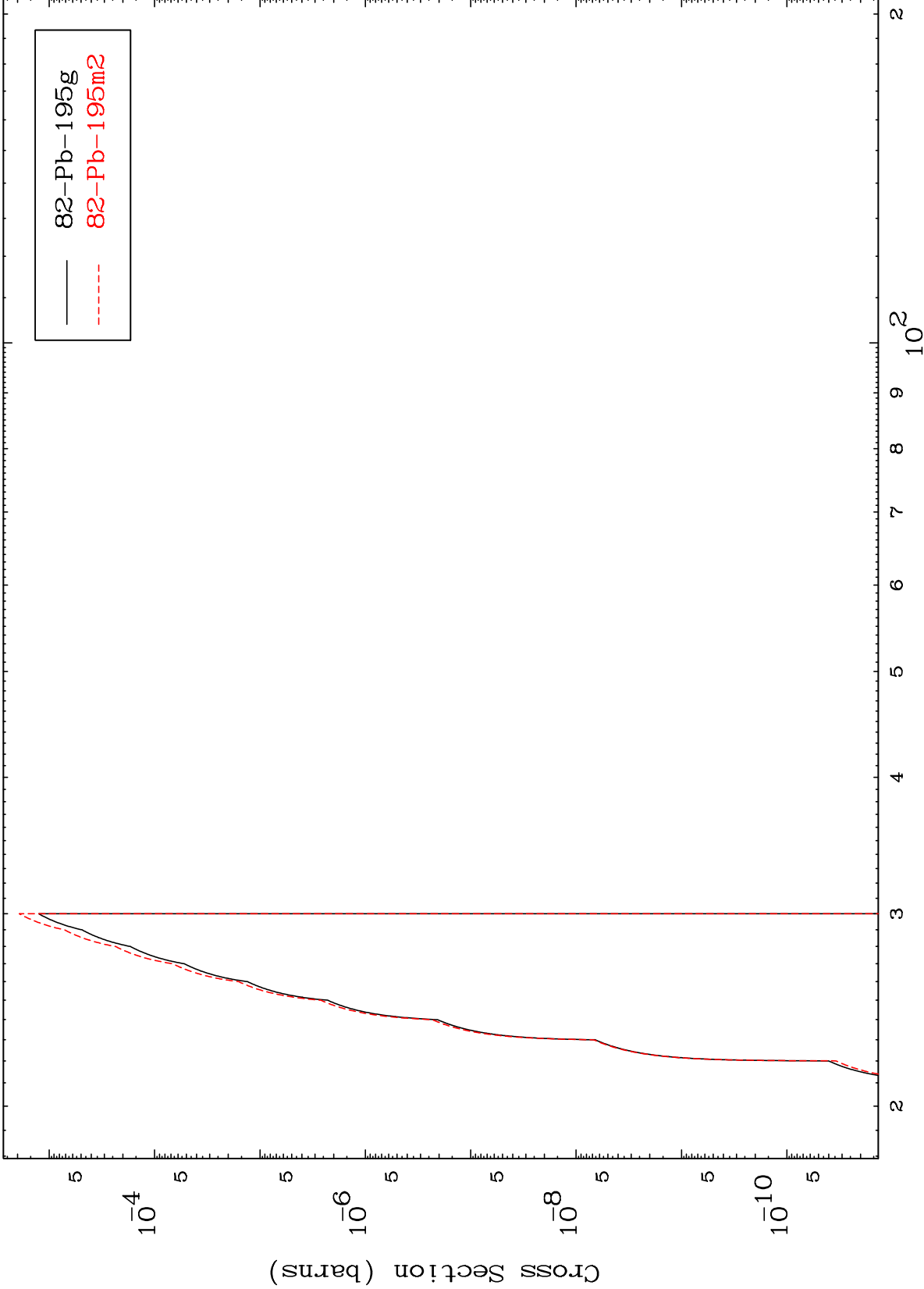
Incident Energy (MeV)

MAT 8294

(n,3n) p

83-Bi-198n

Radionuclide Production Cross Section



34

Incident Energy (MeV)

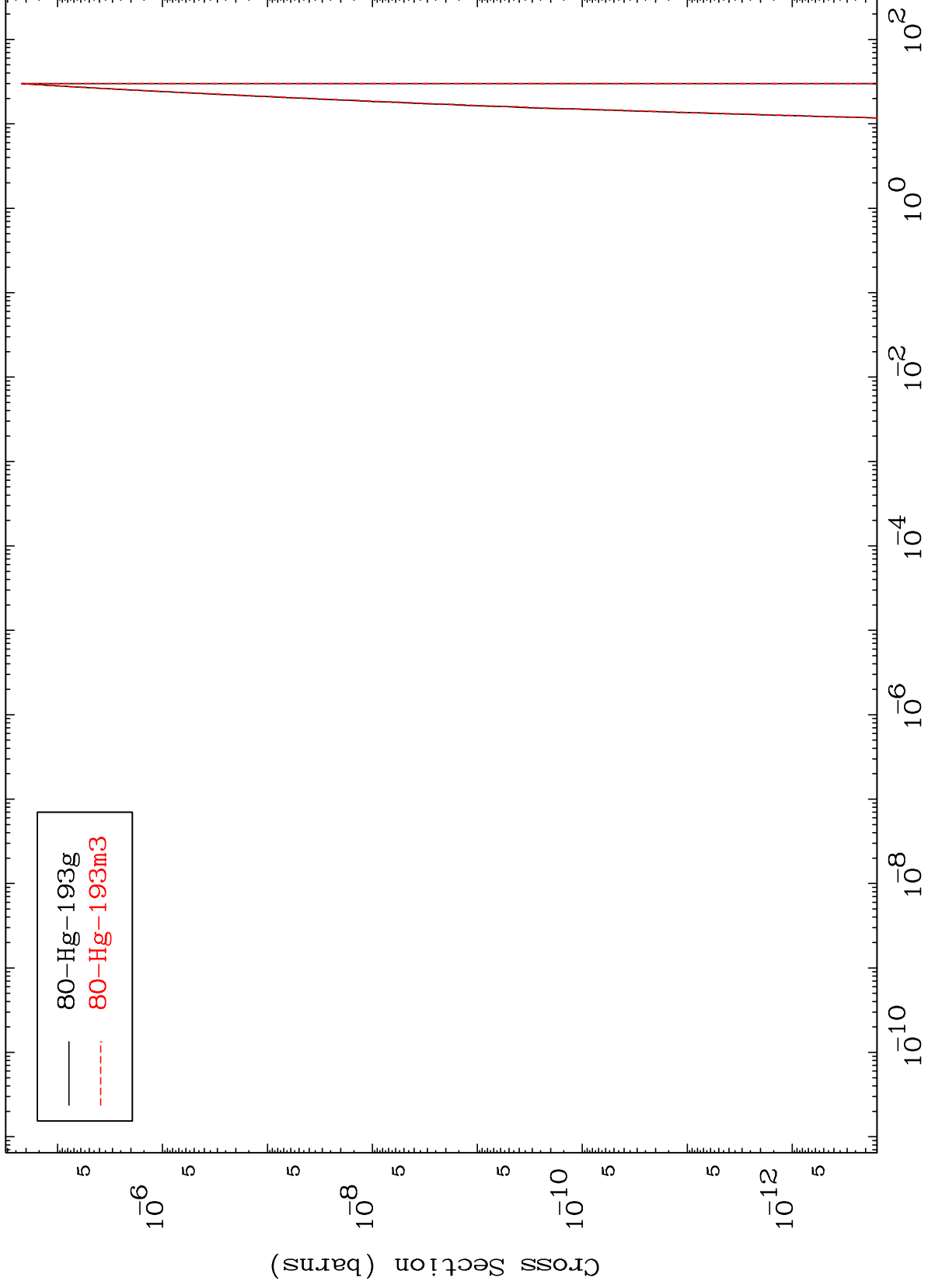
83-Bi-198n

MAT 8294

(n,n') p α

83-Bi-198n

Radionuclide Production Cross Section



35

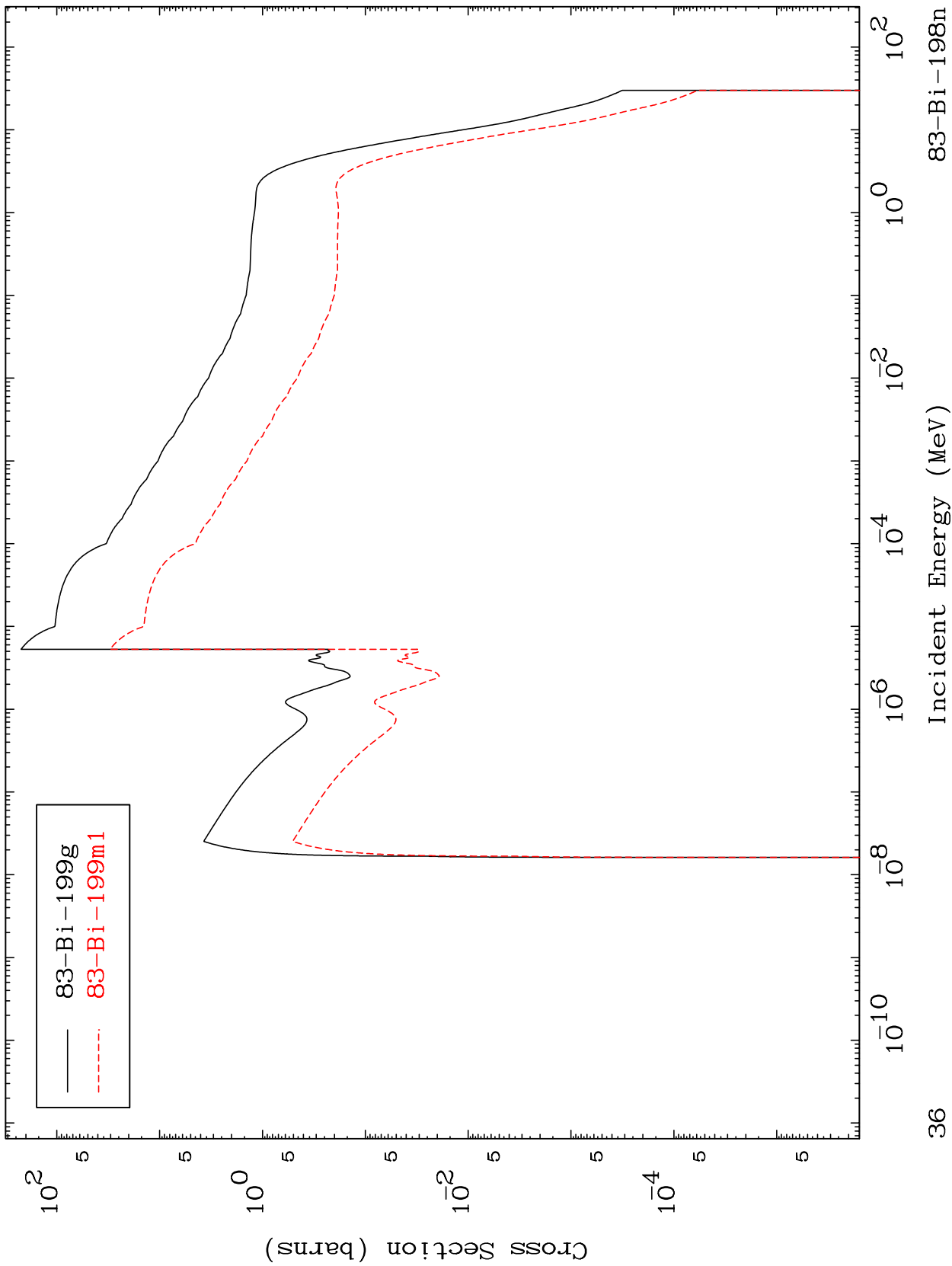
Incident Energy (MeV)

83-Bi-198n

MAT 8294

$^{83}\text{Bi}-198\text{n}$

Radionuclide Production Cross Section
(n, γ)



36

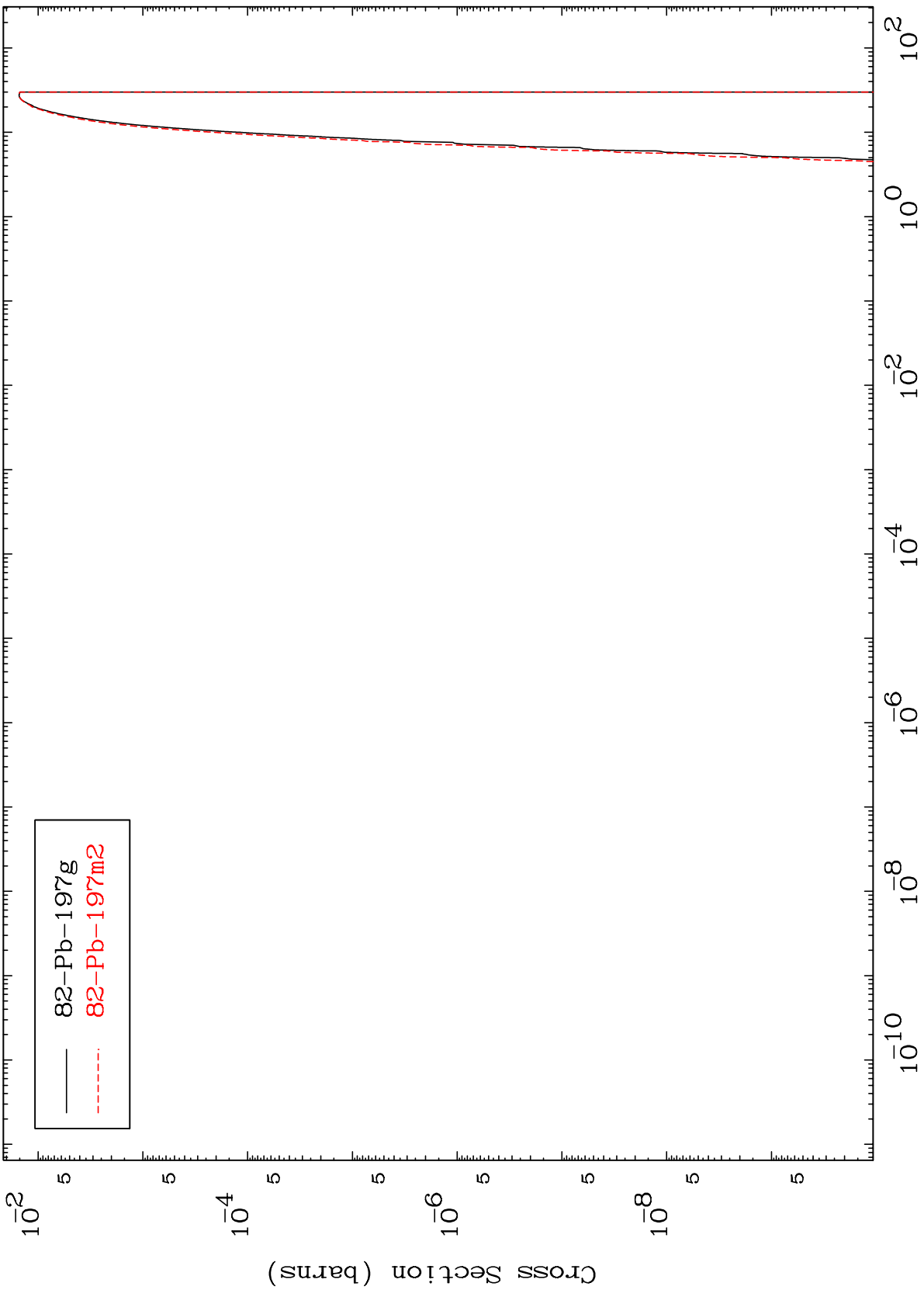
$^{83}\text{Bi}-198\text{n}$

MAT 8294

(n,d)

⁸³Bi-198n

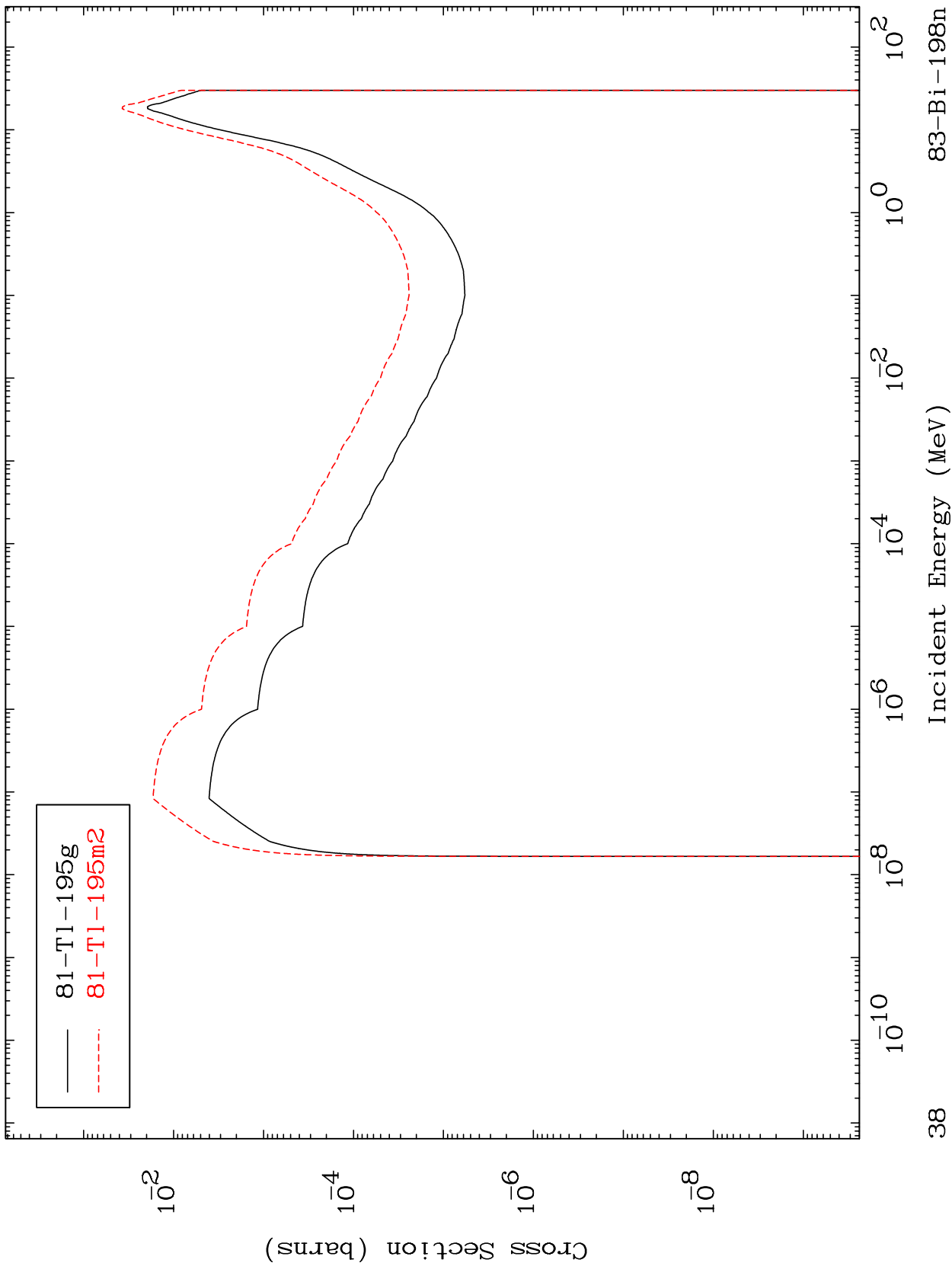
Radionuclide Production Cross Section



MAT 8294

83-Bi-198n

Radionuclide Production Cross Section
(n, α)

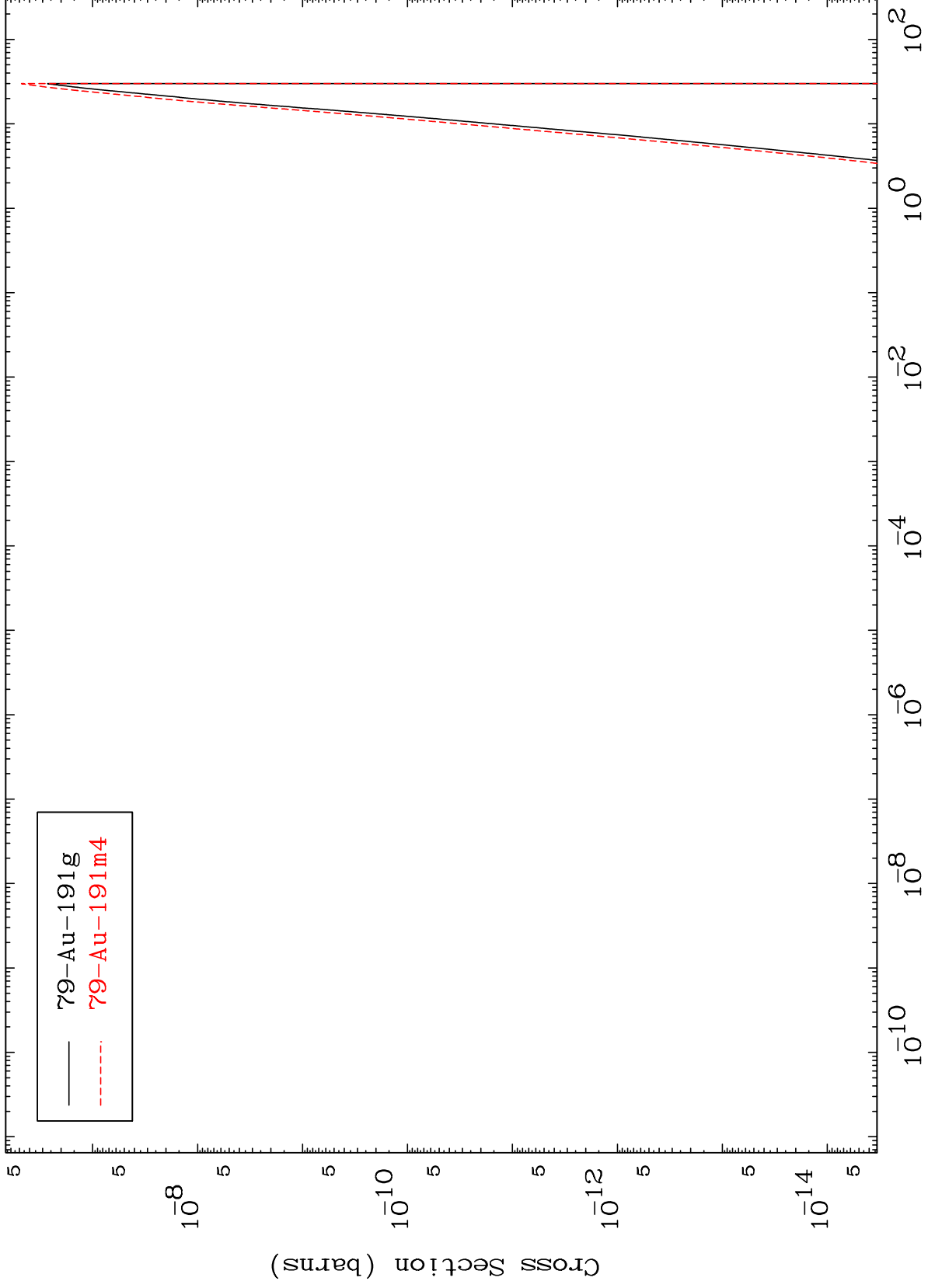


MAT 8294

(n,2α)

83-Bi-198n

Radionuclide Production Cross Section



39

Incident Energy (MeV)

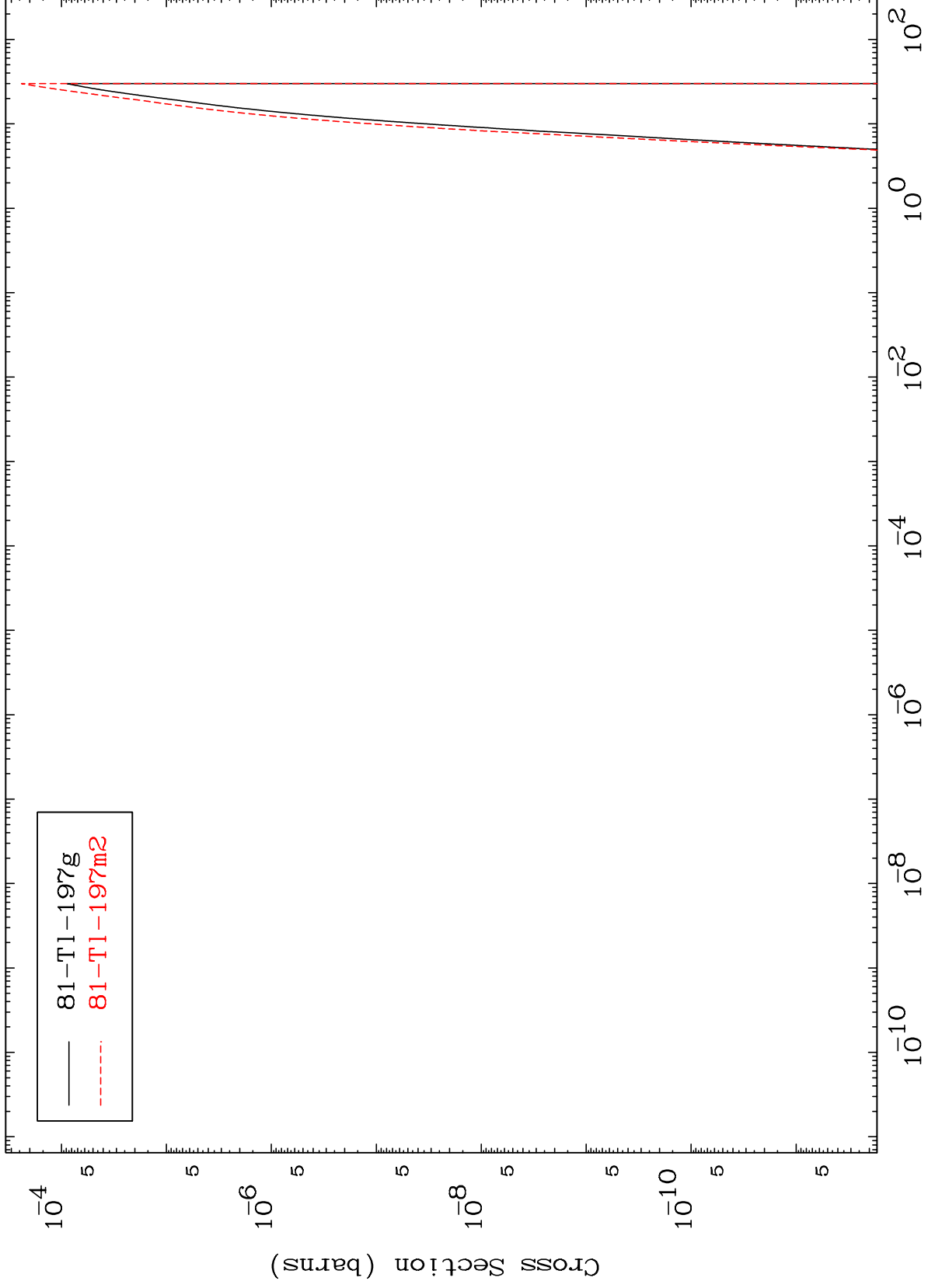
83-Bi-198n

MAT 8294

(n,2p)

⁸³Bi-198n

Radionuclide Production Cross Section



40

Incident Energy (MeV)

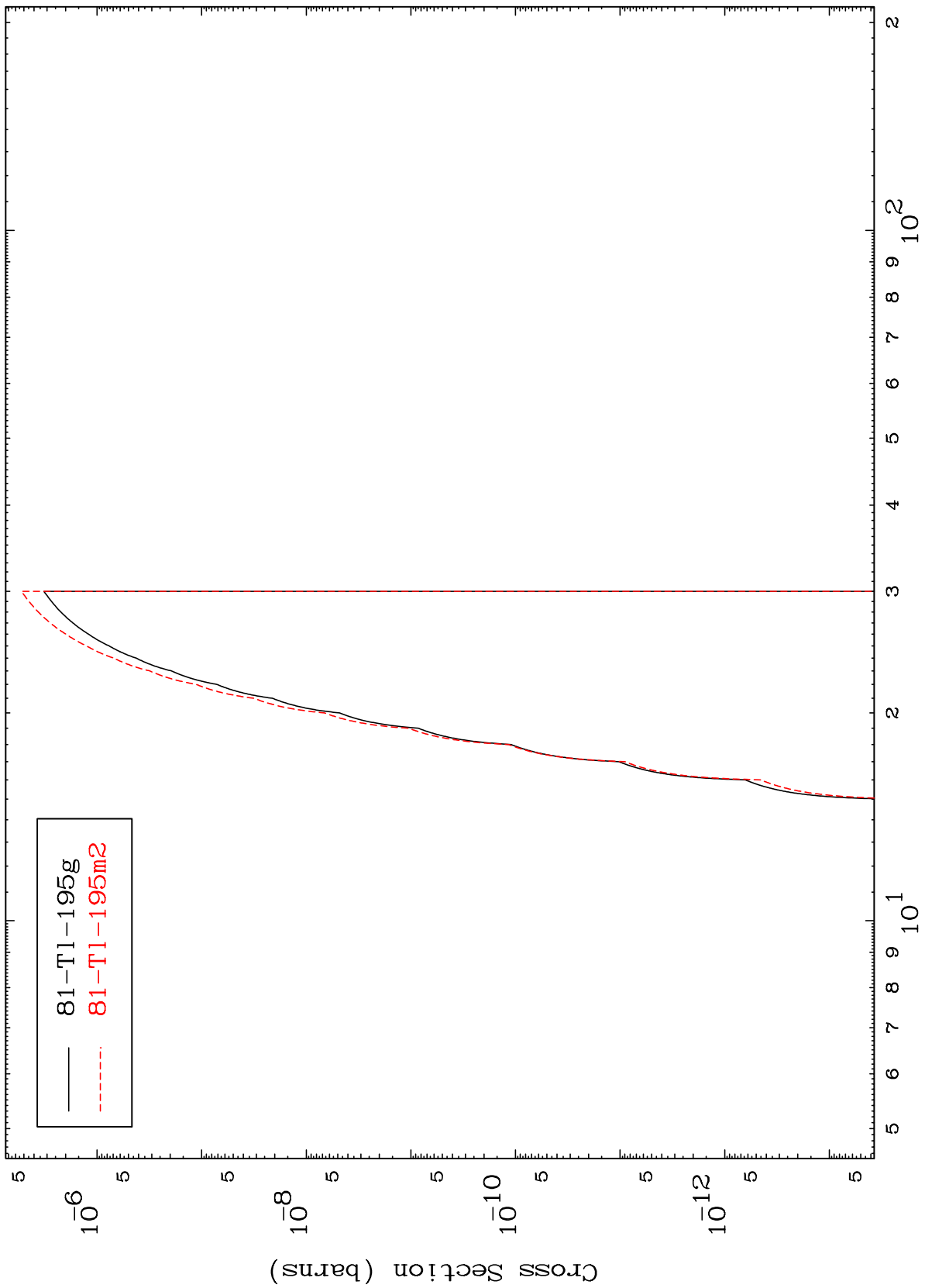
⁸³Bi-198n

MAT 8294

(n,p) t

83-Bi-198n

Radionuclide Production Cross Section



41

Incident Energy (MeV)

83-Bi-198n

MAT 8294

(n,d) α

83-Bi-198n

Radionuclide Production Cross Section

