

Program EVALPLOT
(Version 2021-1)

by

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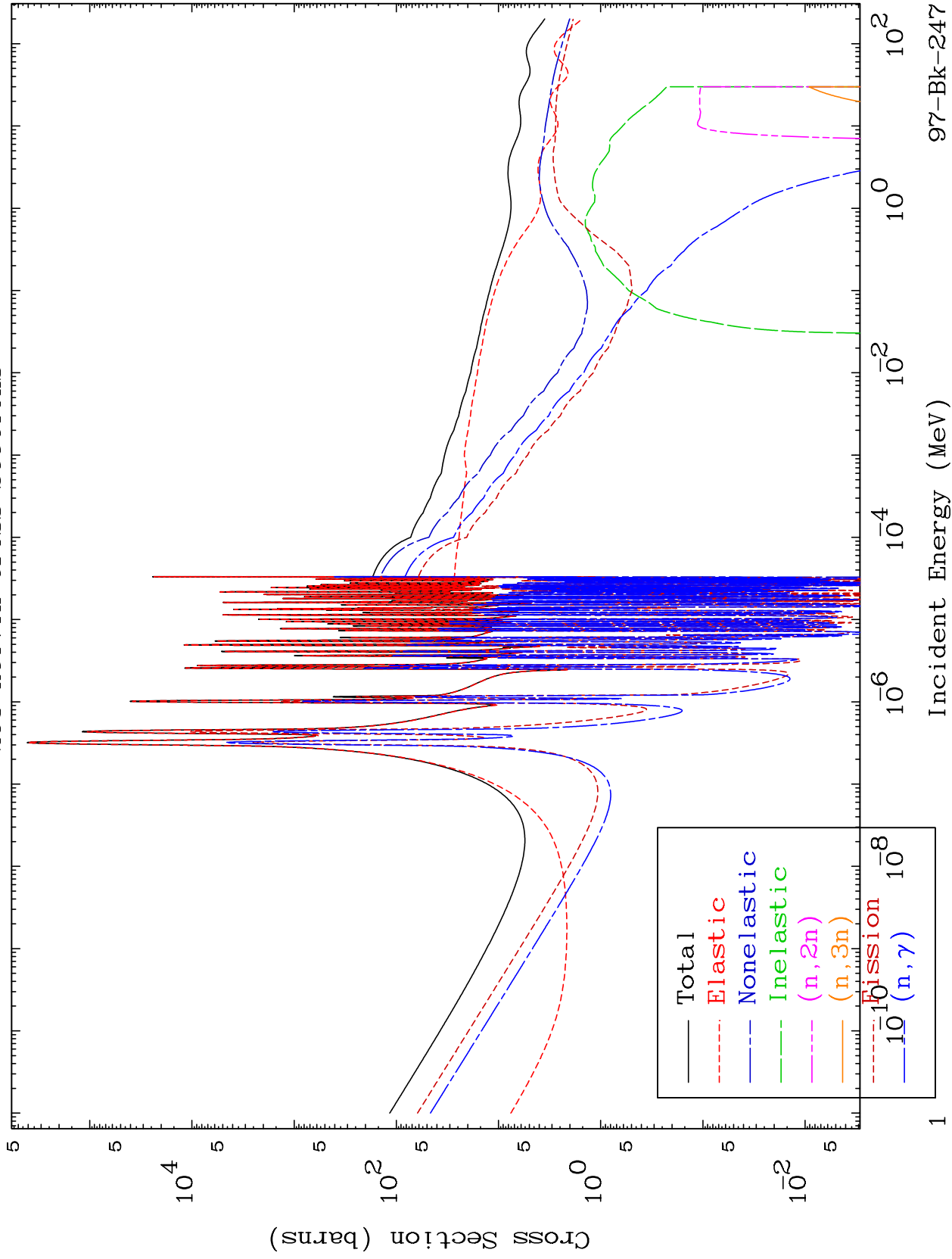
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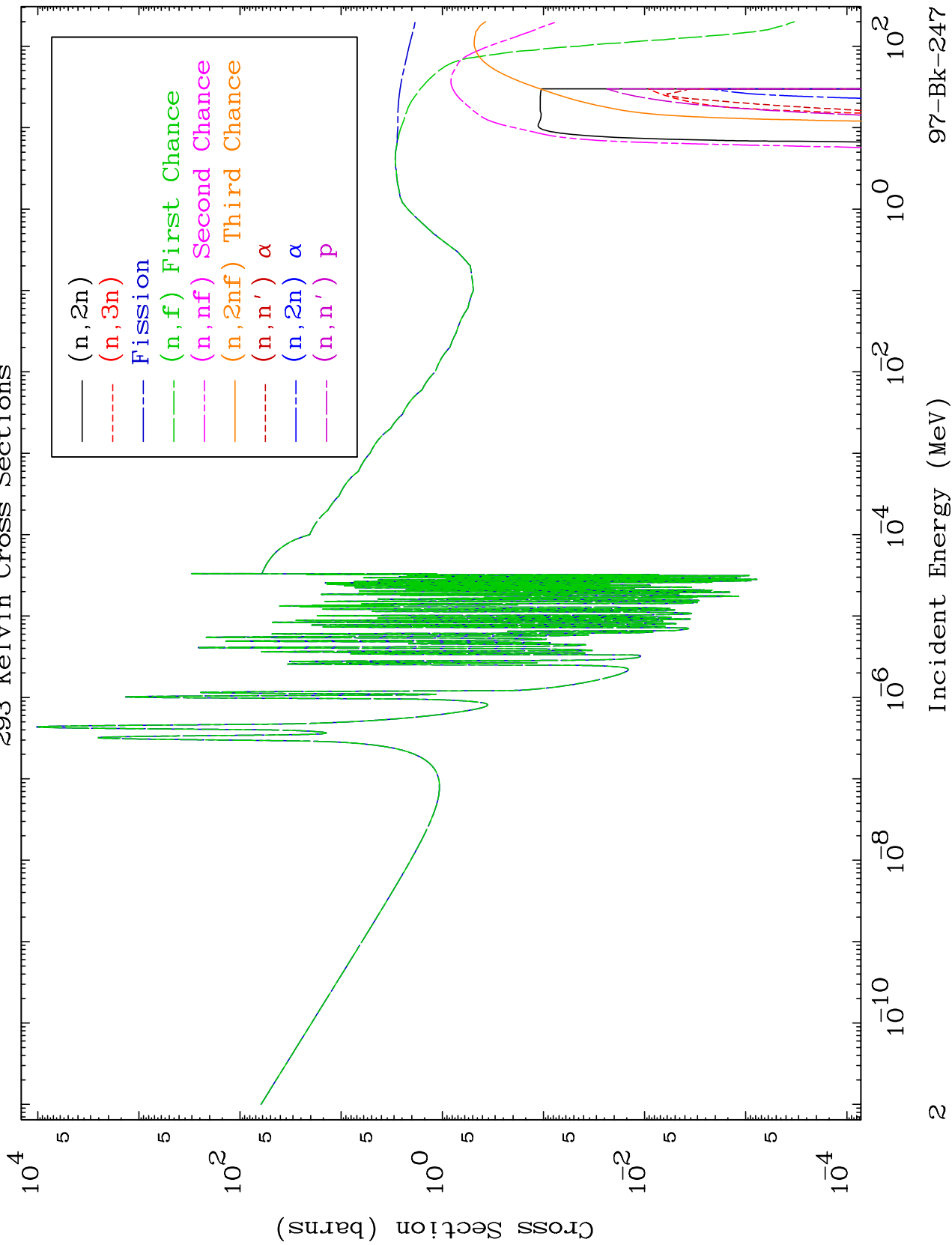
Press Mouse Button to Start

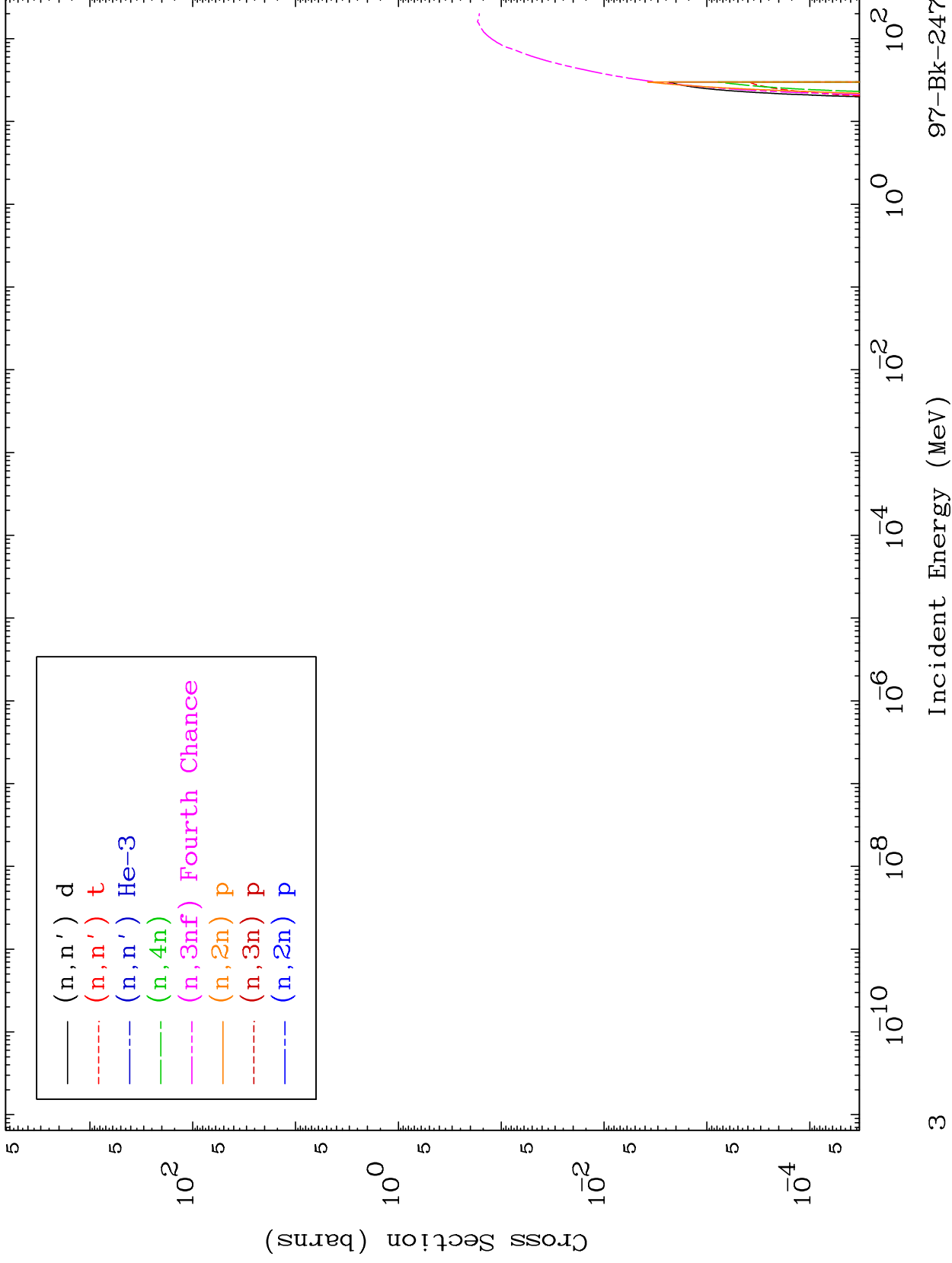
MAT 9746

Neutron Major
293 Kelvin Cross Sections

97-Bk-247



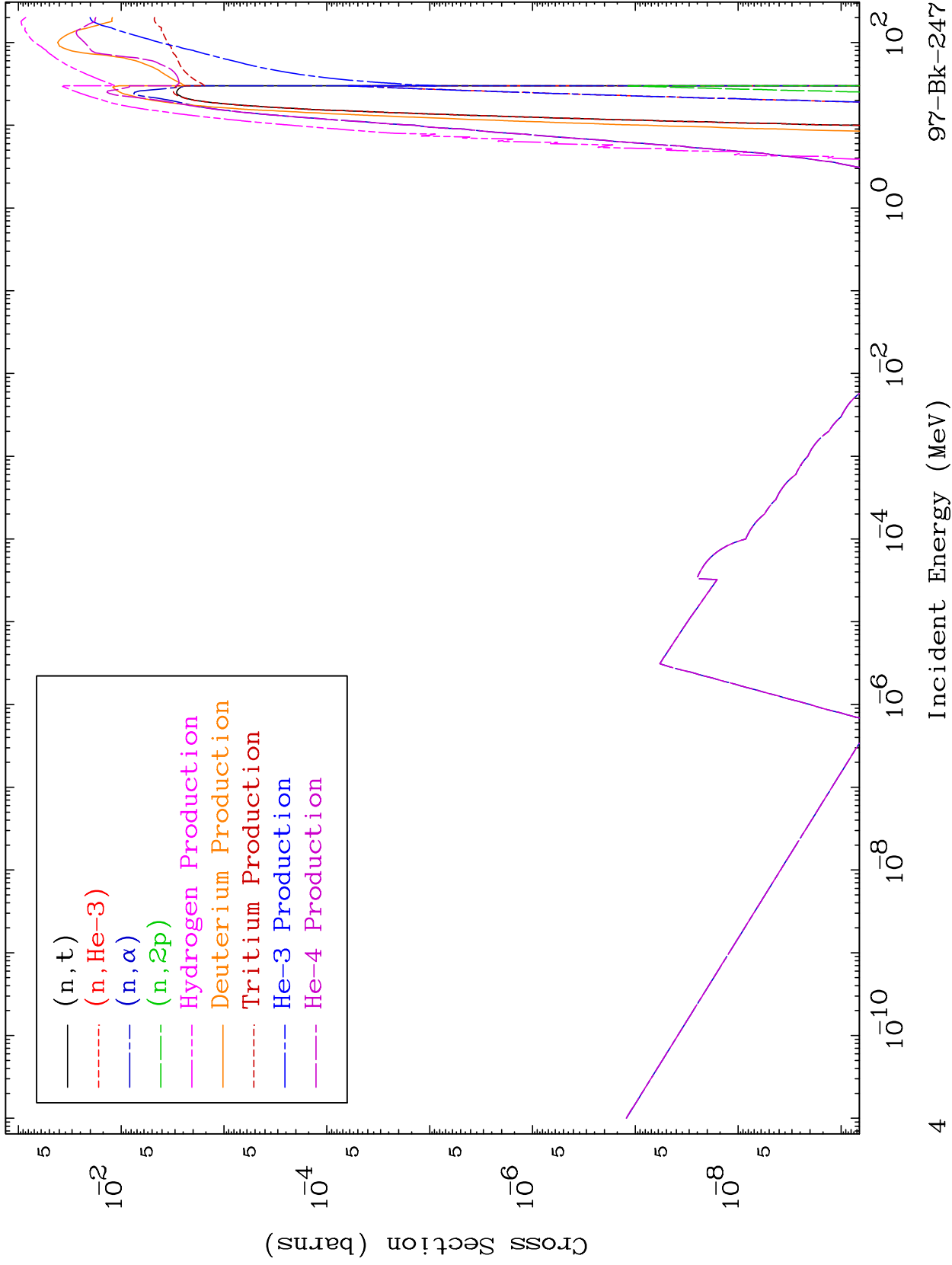




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Neutron Absorption
293 Kelvin Cross Sections

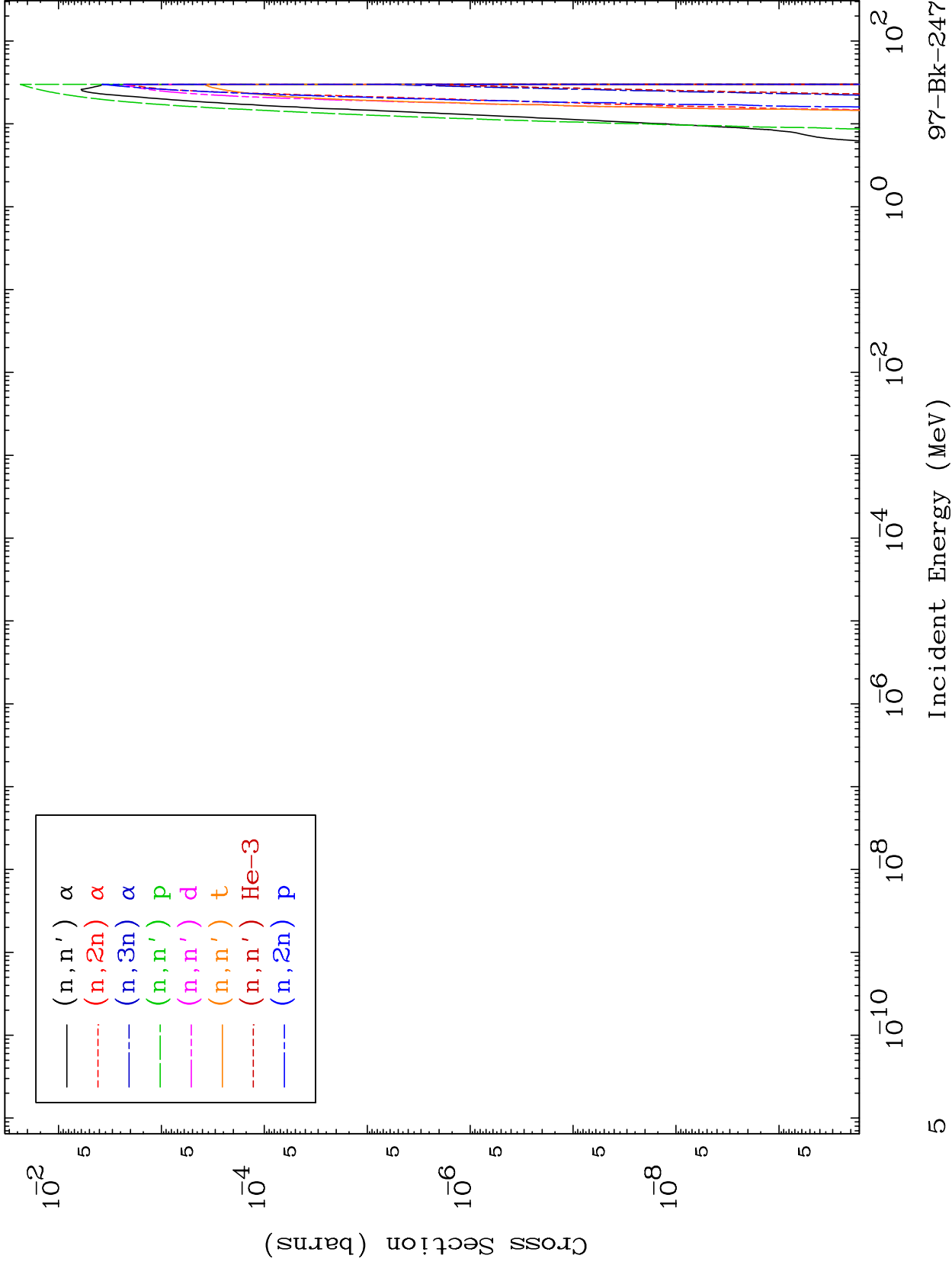
97-Bk-247



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Charged Particle
293 Kelvin Cross Sections

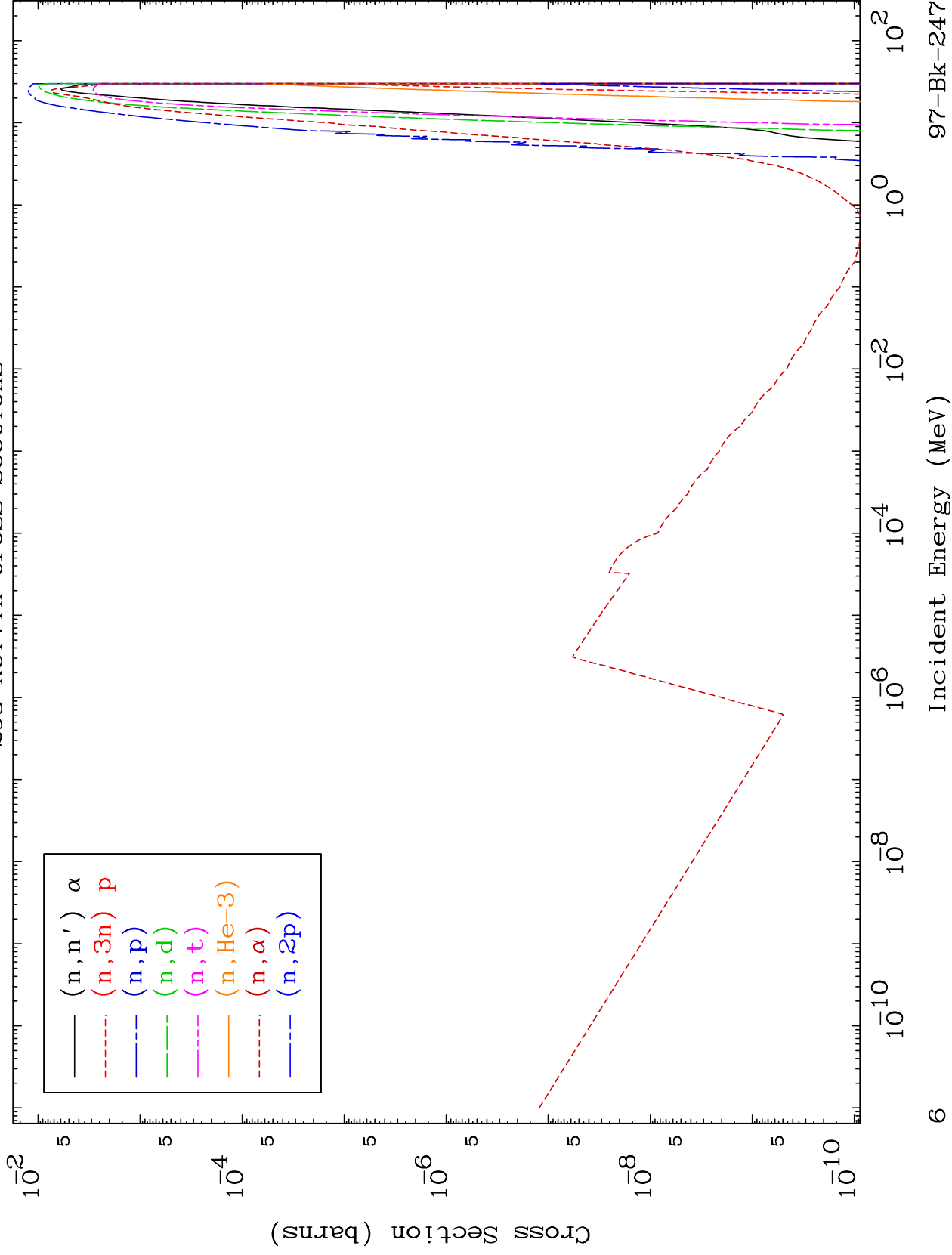
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Charged Particle
293 Kelvin Cross Sections

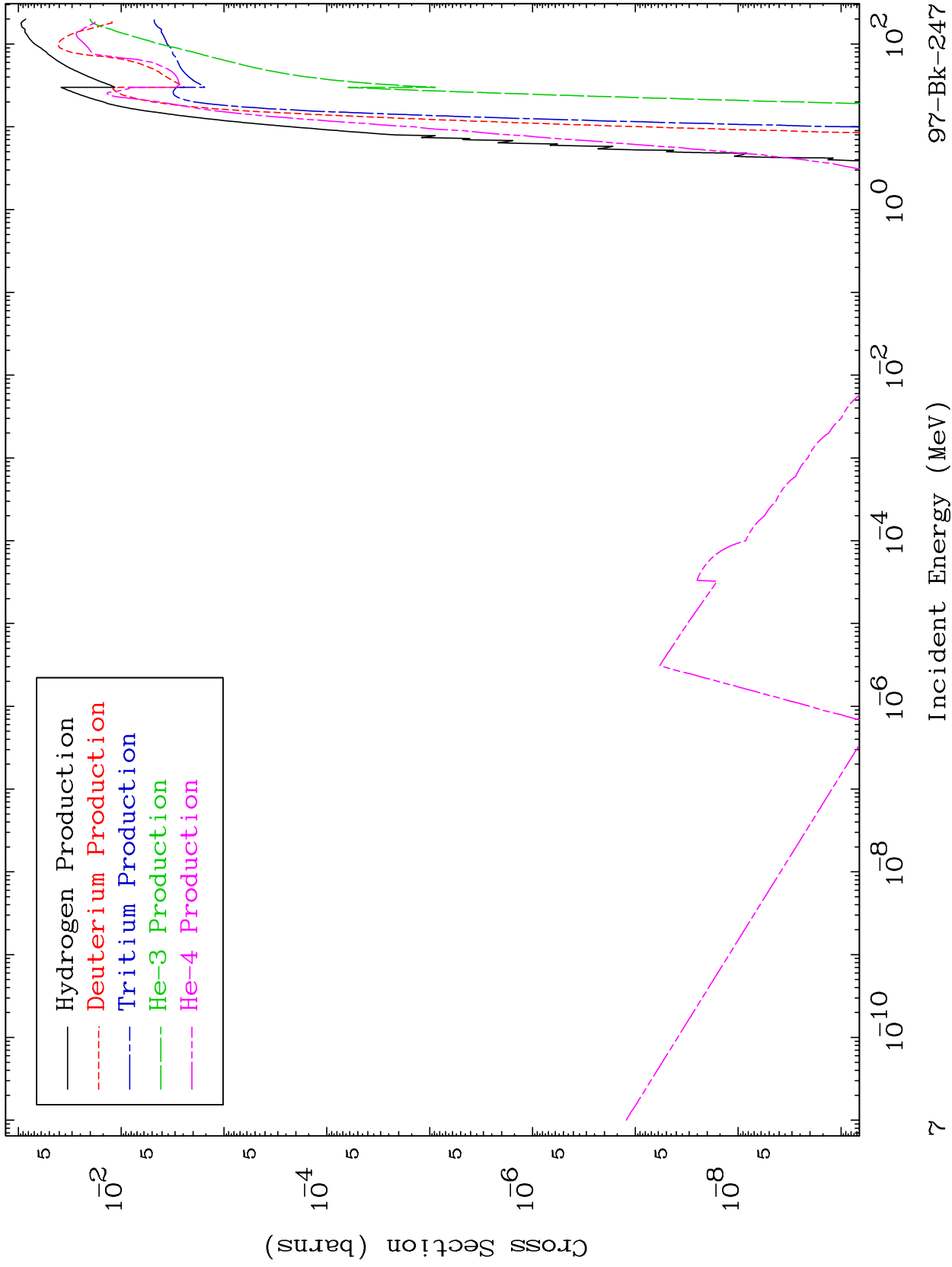
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Particle Production
293 Kelvin Cross Sections

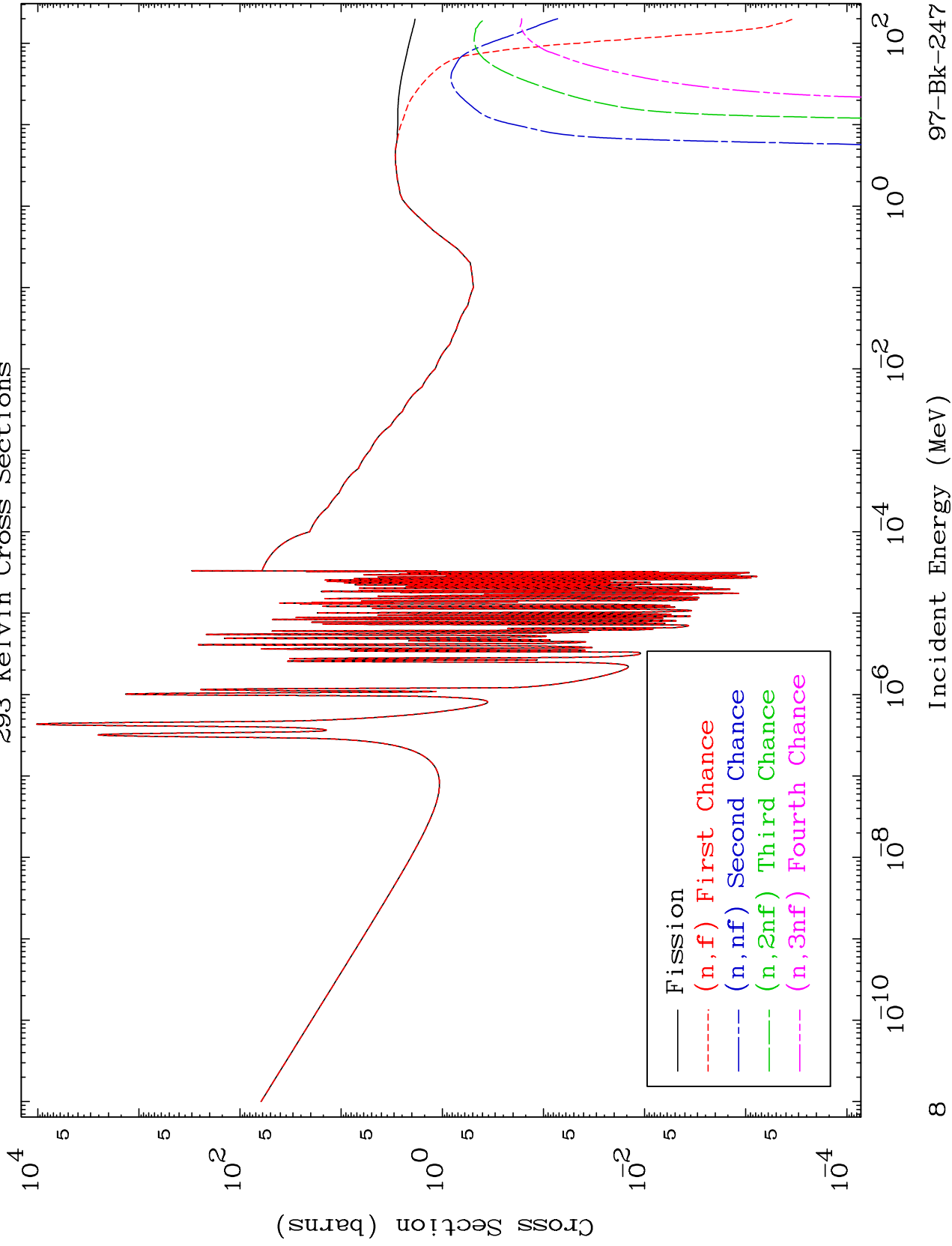
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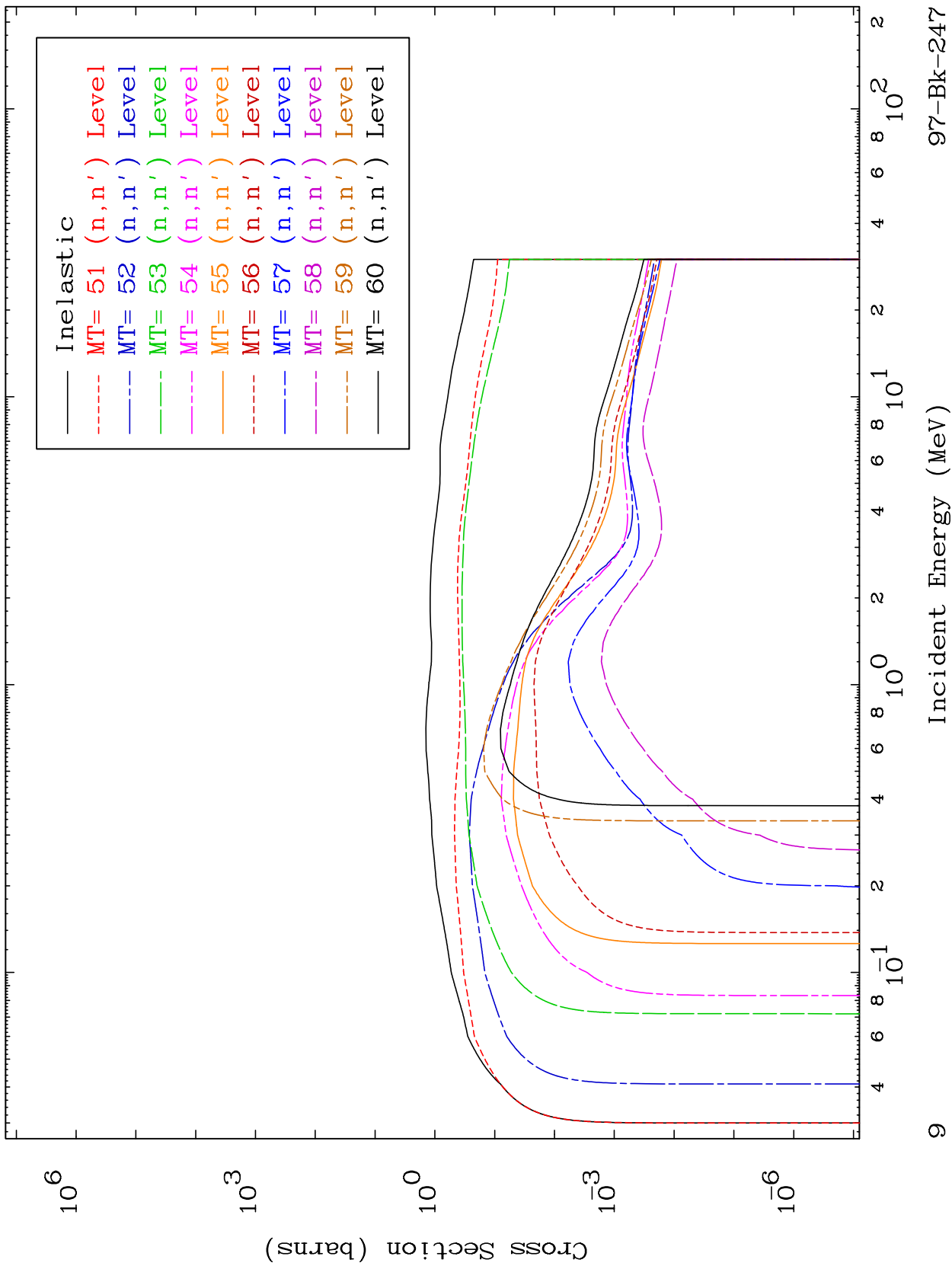


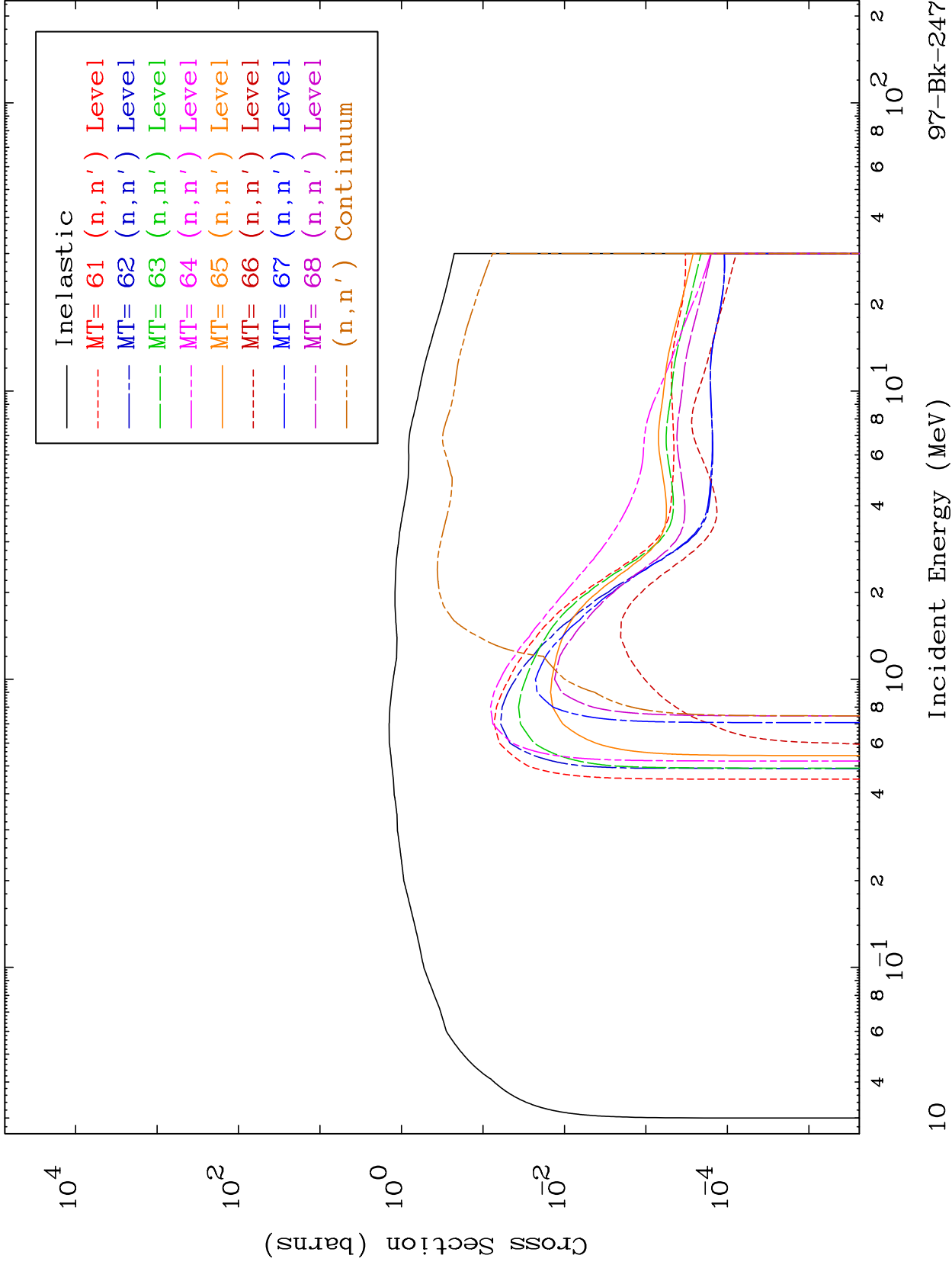
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Fission
293 Kelvin Cross Sections

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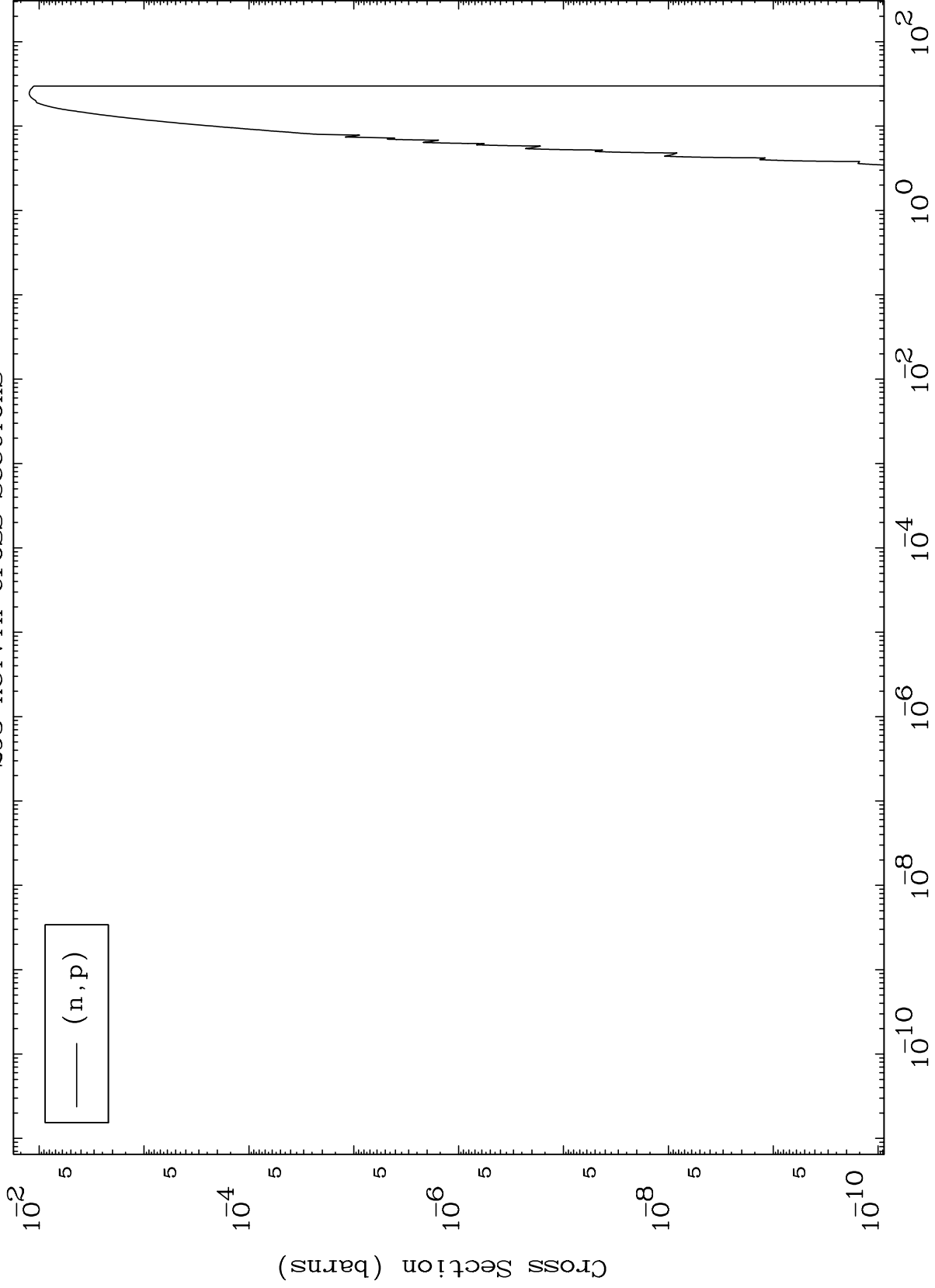




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(n,p) Levels
293 Kelvin Cross Sections

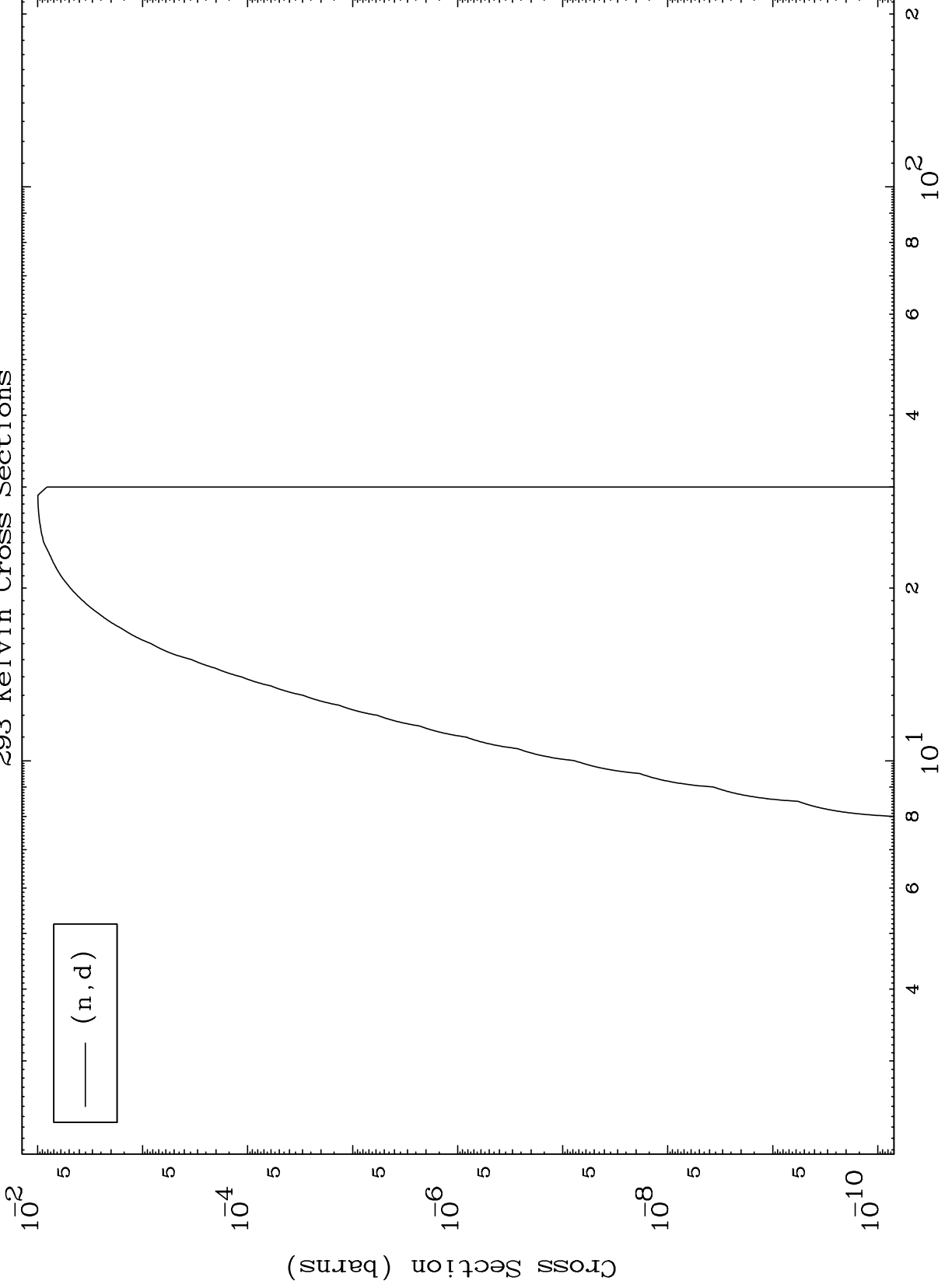
97-Bk-247



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(n,d) Levels
293 Kelvin Cross Sections

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12

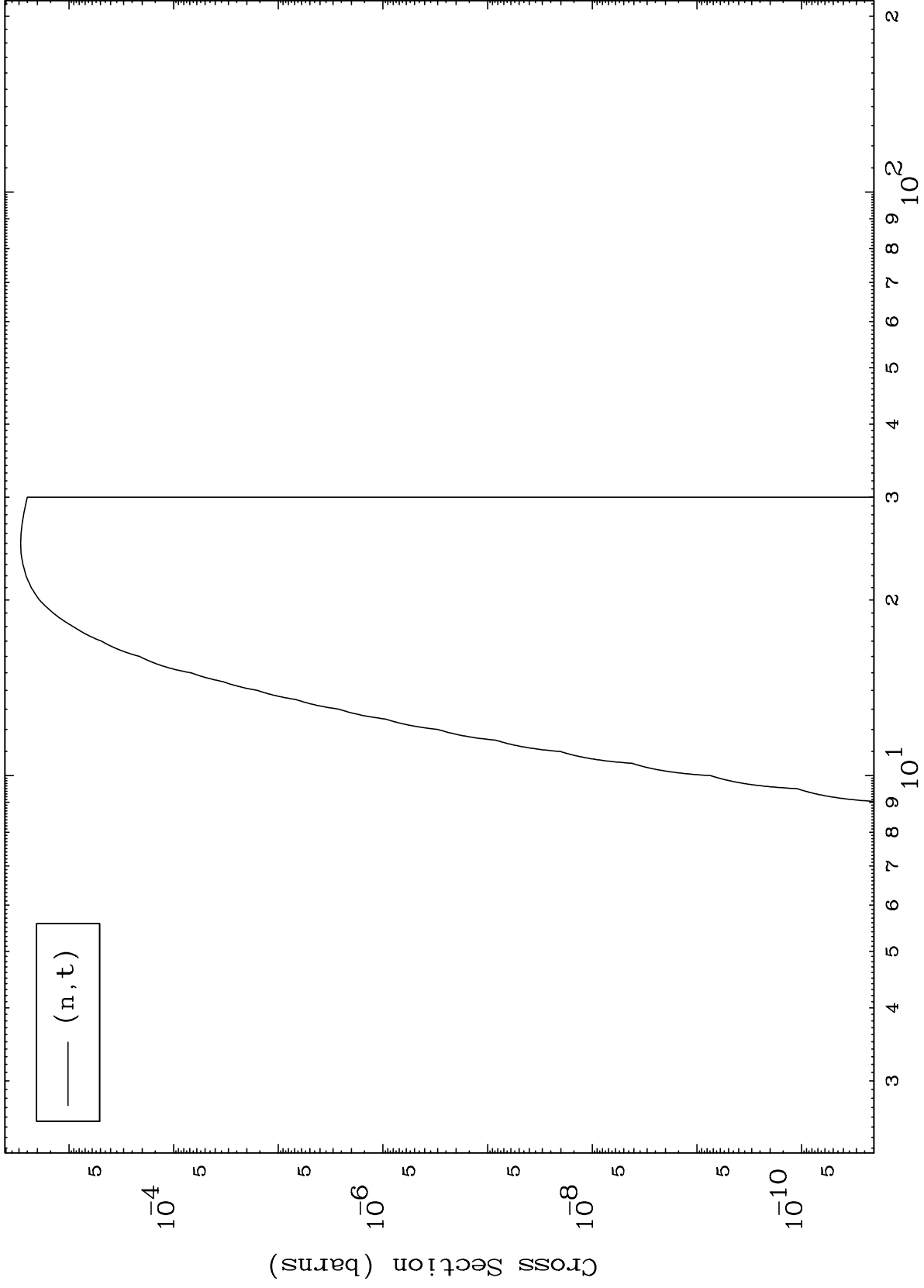
Incident Energy (MeV)

97-Bk-247

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(n,t) Levels
293 Kelvin Cross Sections

97-Bk-247



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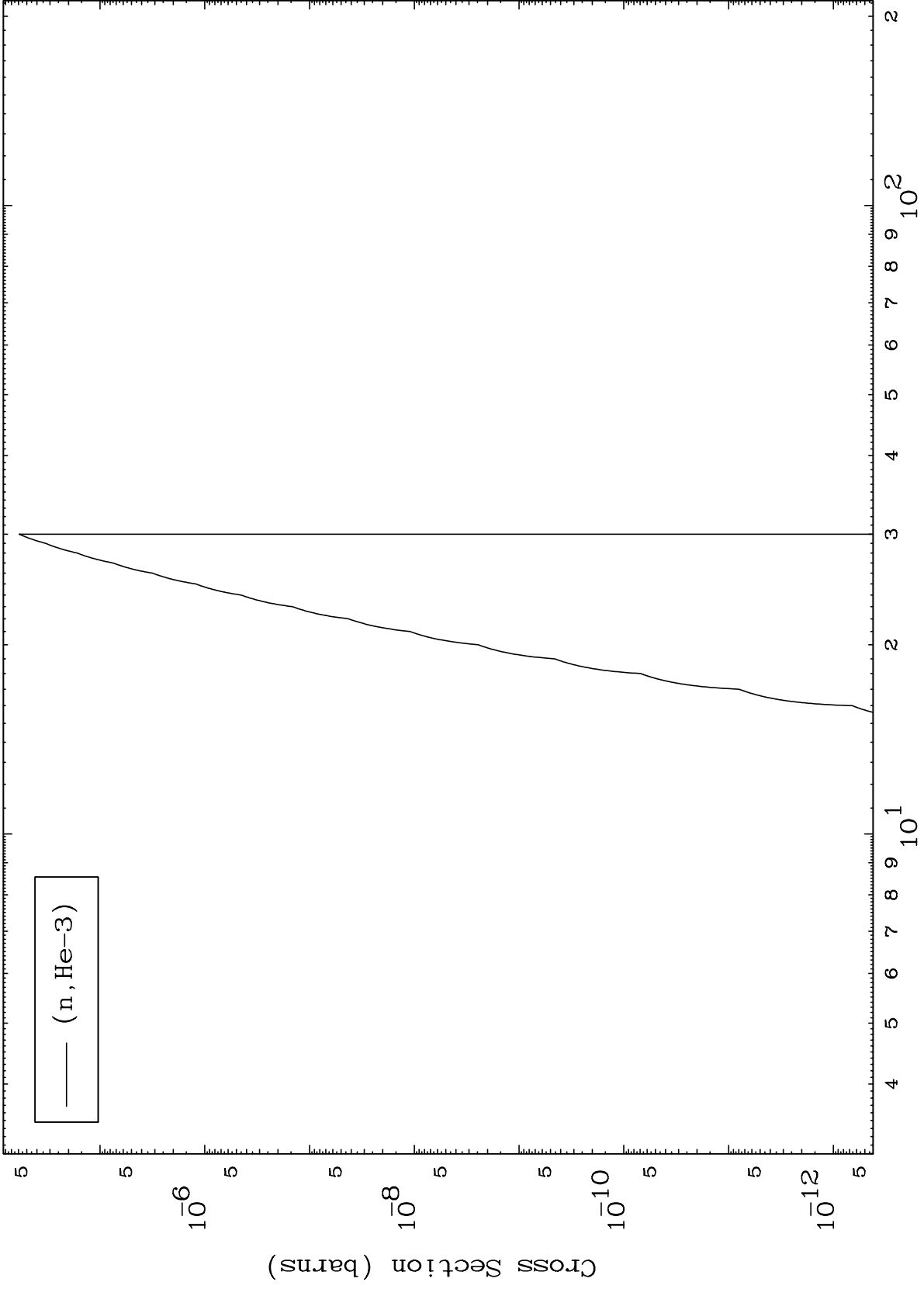
Incident Energy (MeV)

97-Bk-247

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(n,He3) Levels
293 Kelvin Cross Sections

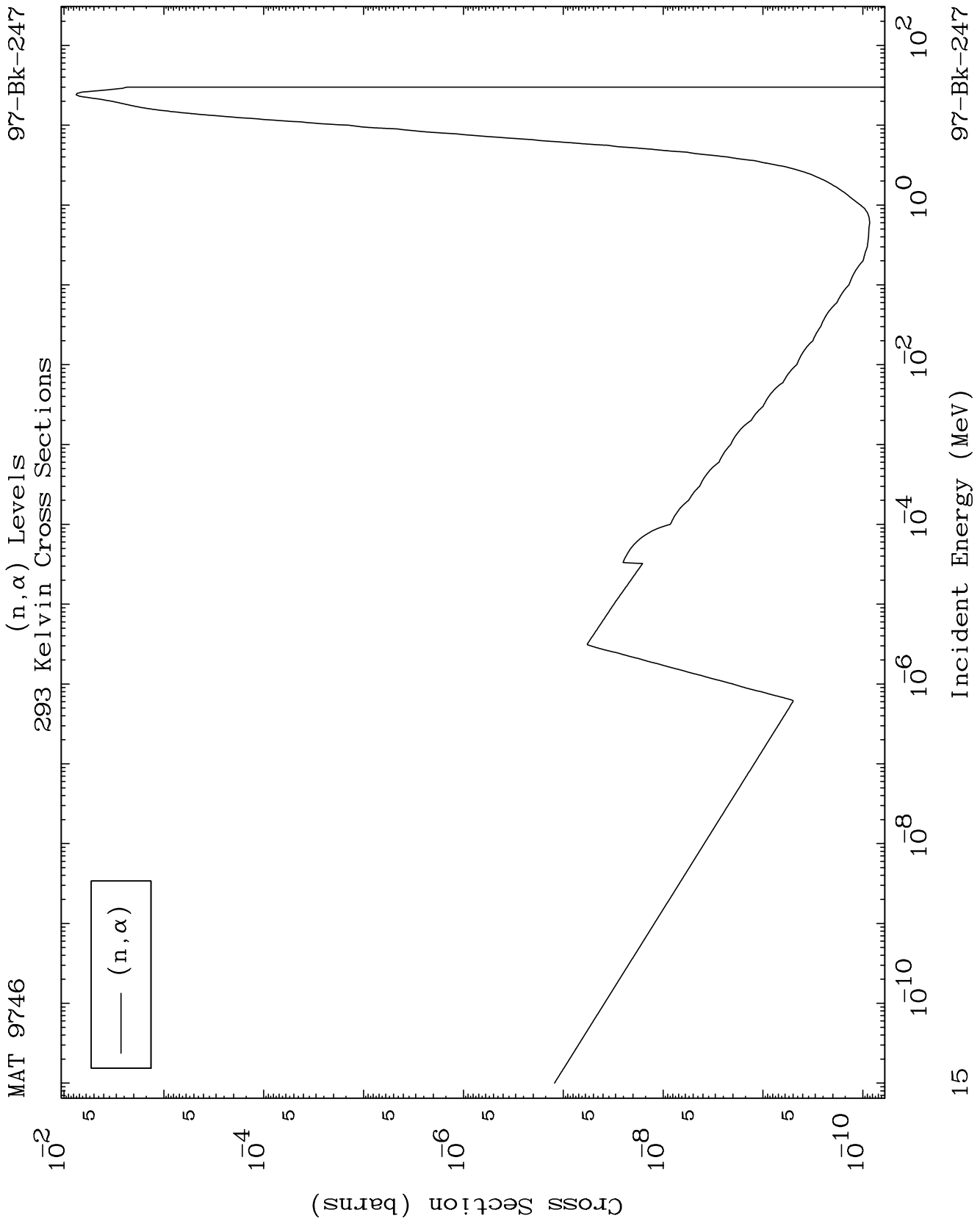
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Incident Energy (MeV)

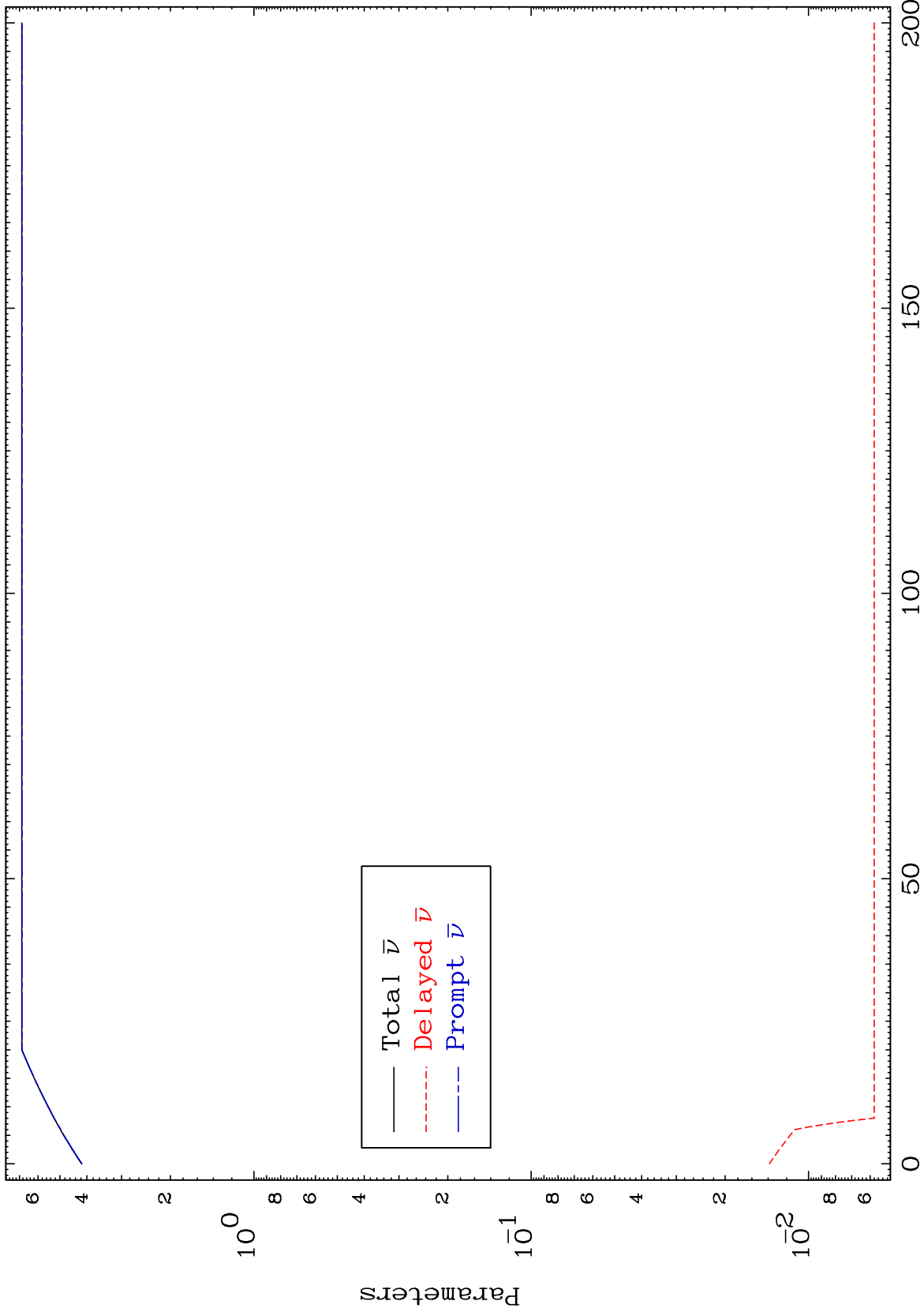
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Energy Release
Parameters

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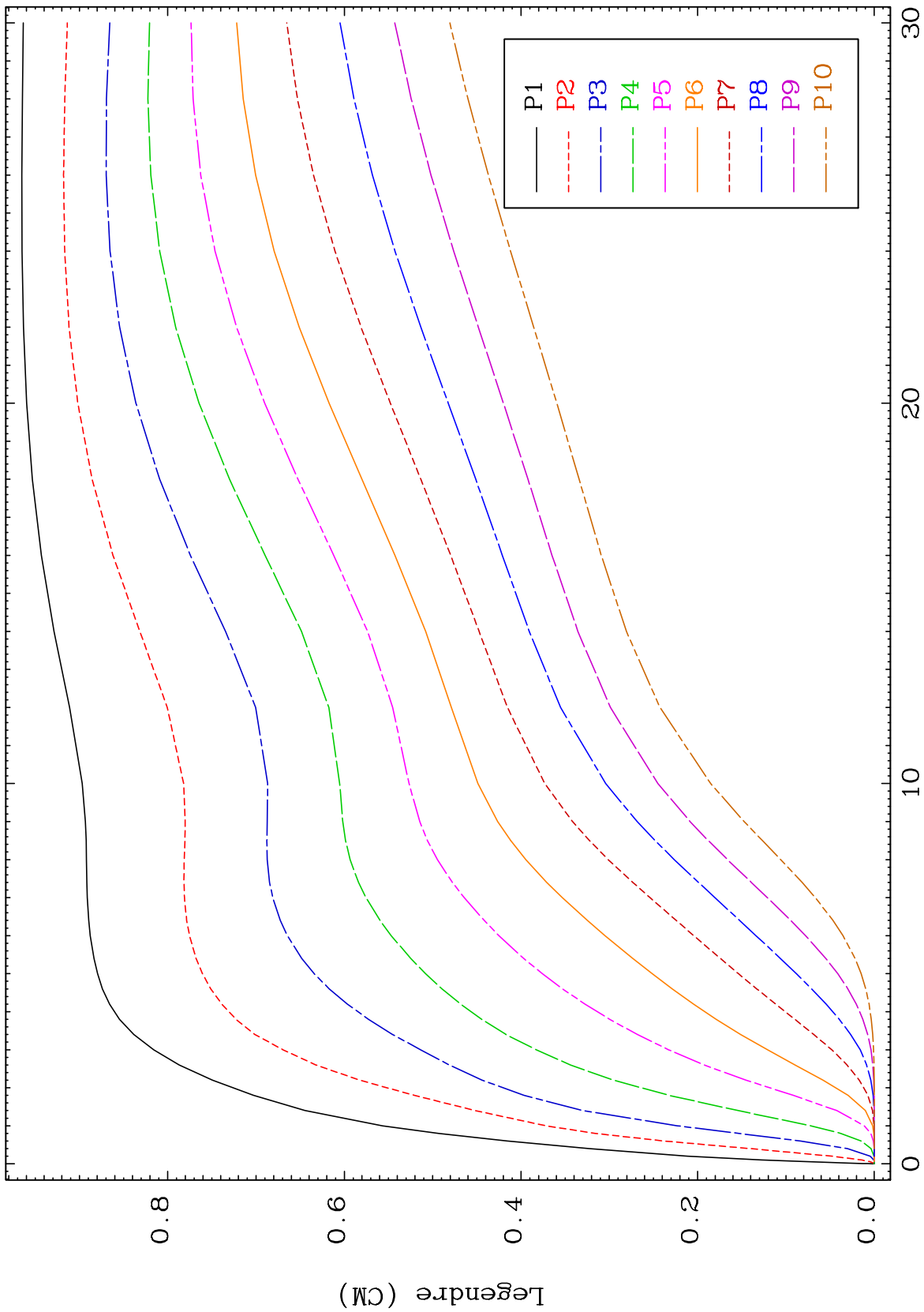
Incident Energy (MeV)

97-Bk-247

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97-Bk-247

Elastic Legendre Coefficients



97-Bk-247

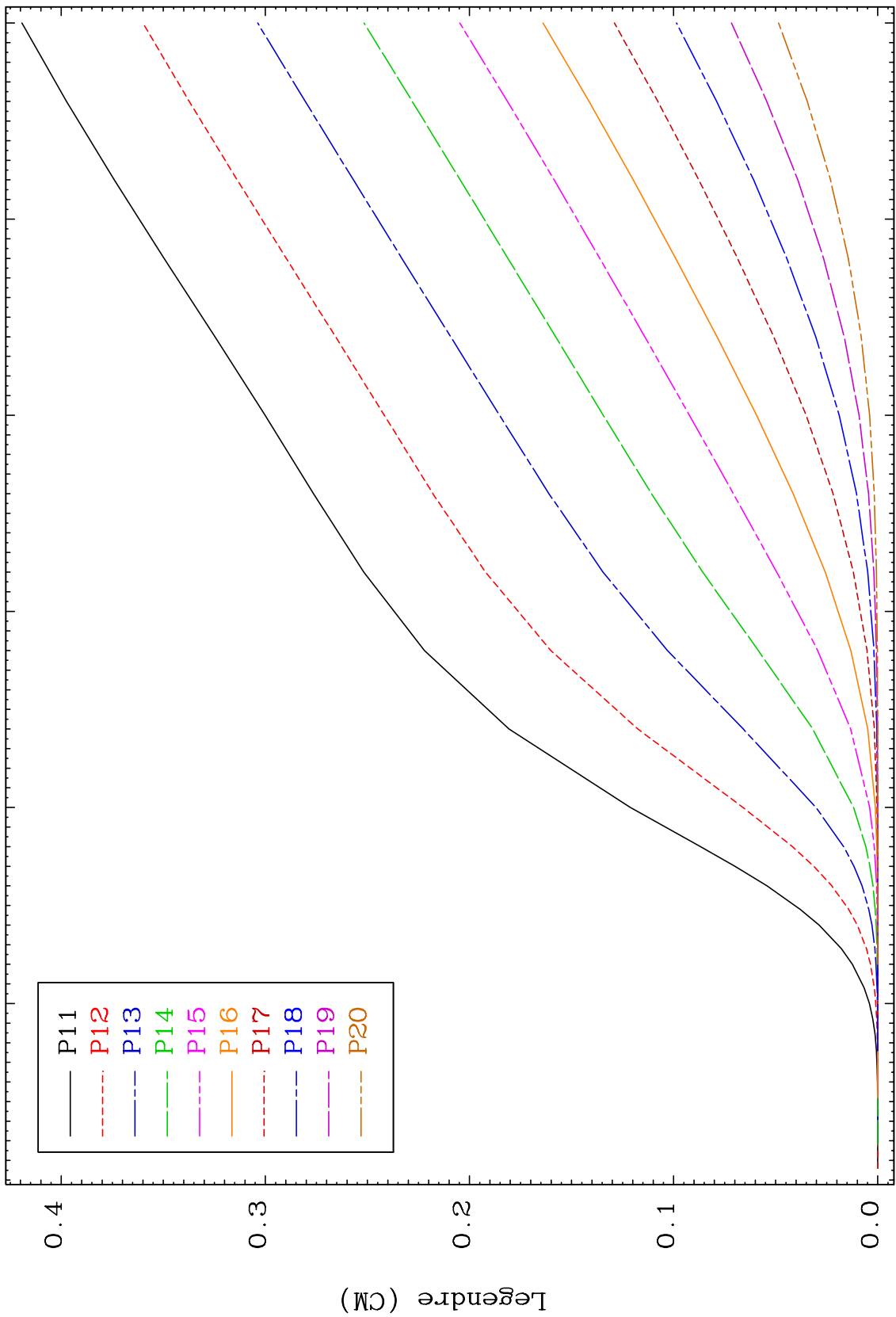
Incident Energy (MeV)

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Elastic Legendre Coefficients

97-Bk-247



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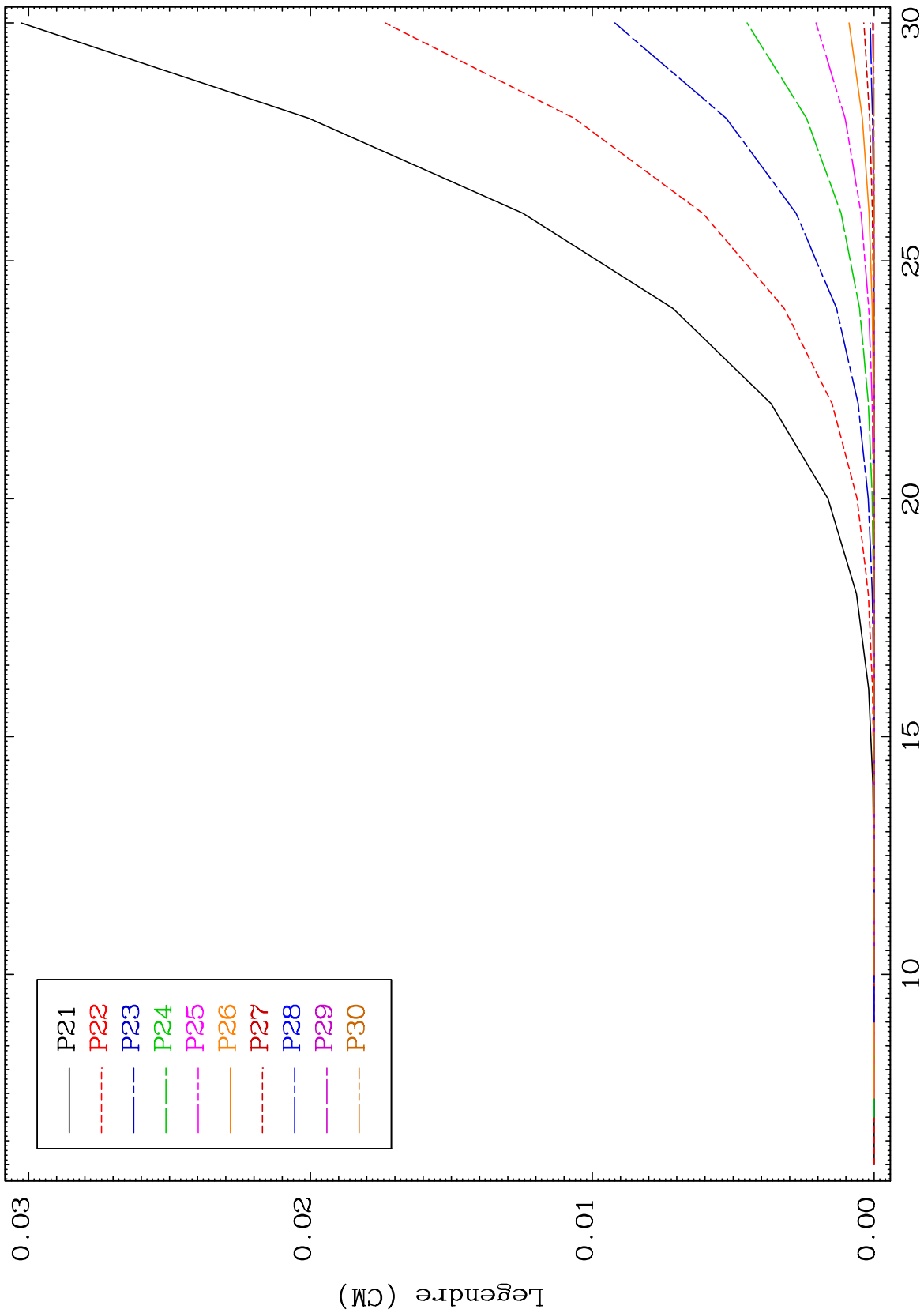
97-Bk-247

Incident Energy (MeV)

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Elastic Legendre Coefficients

97-Bk-247



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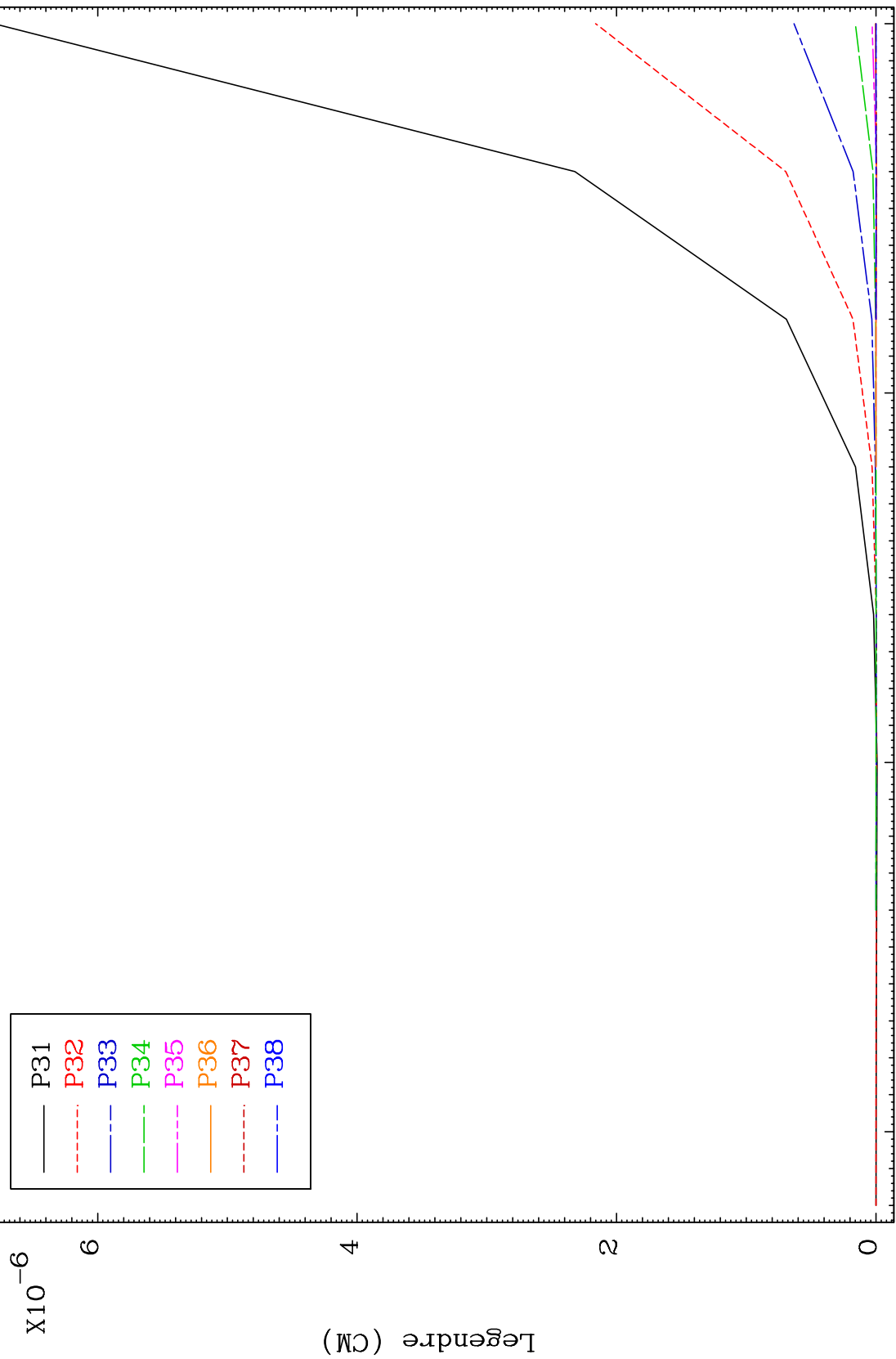
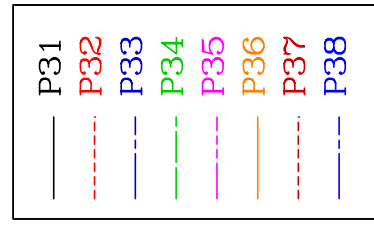
Incident Energy (MeV)

97-Bk-247

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Elastic Legendre Coefficients

97-Bk-247

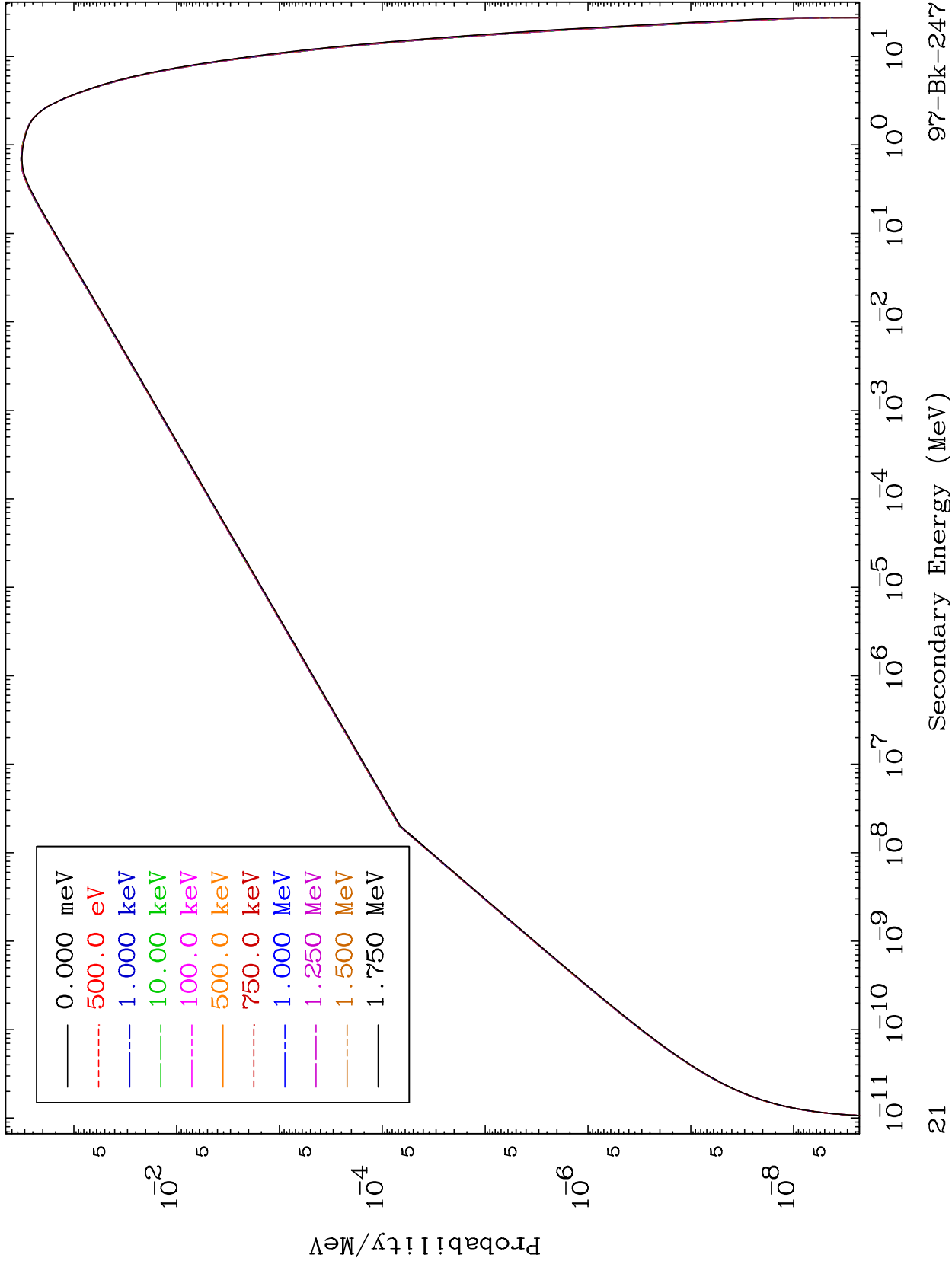


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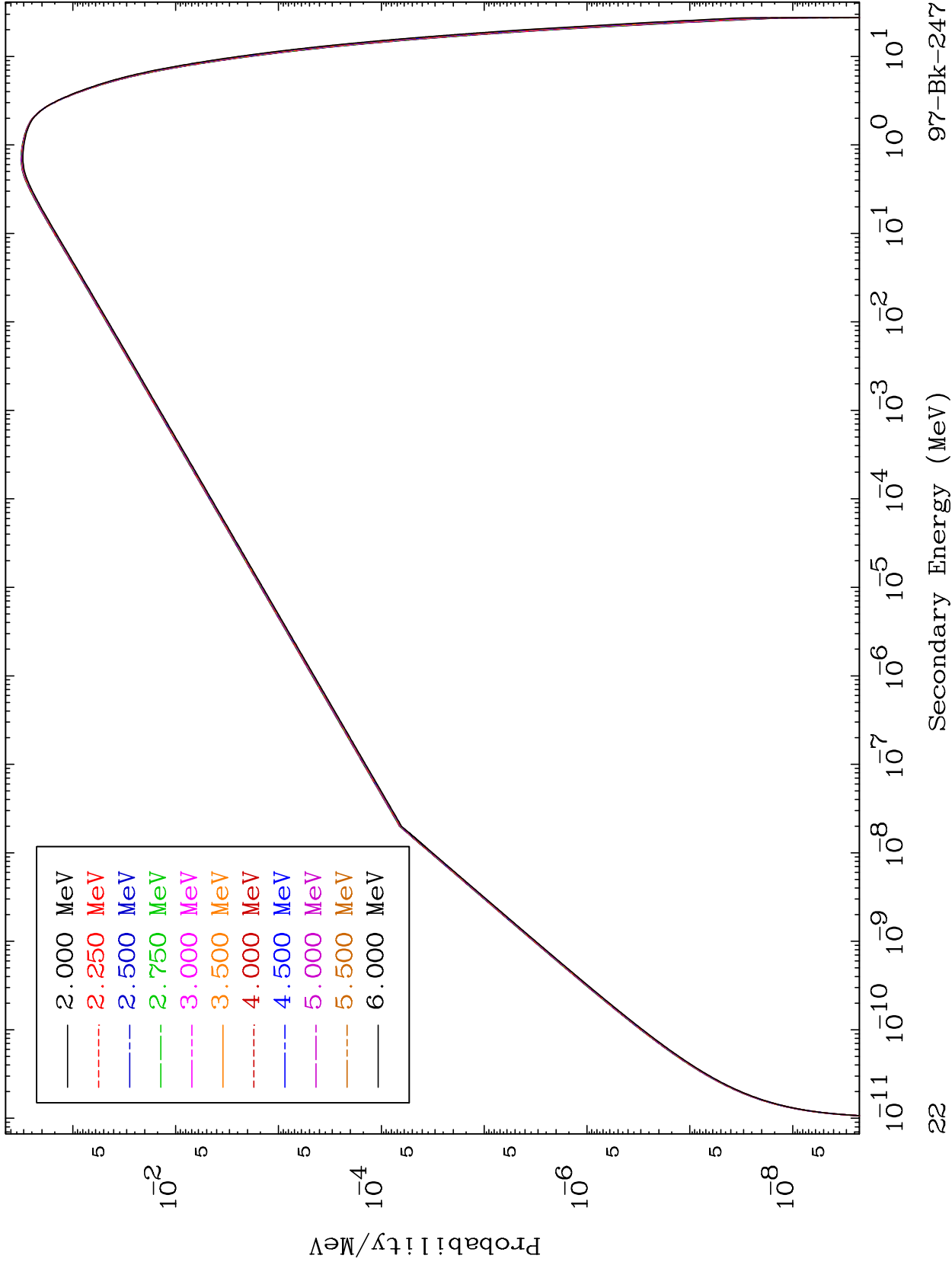
Incident Energy (MeV)

97-Bk-247

Fission Energy Distributions



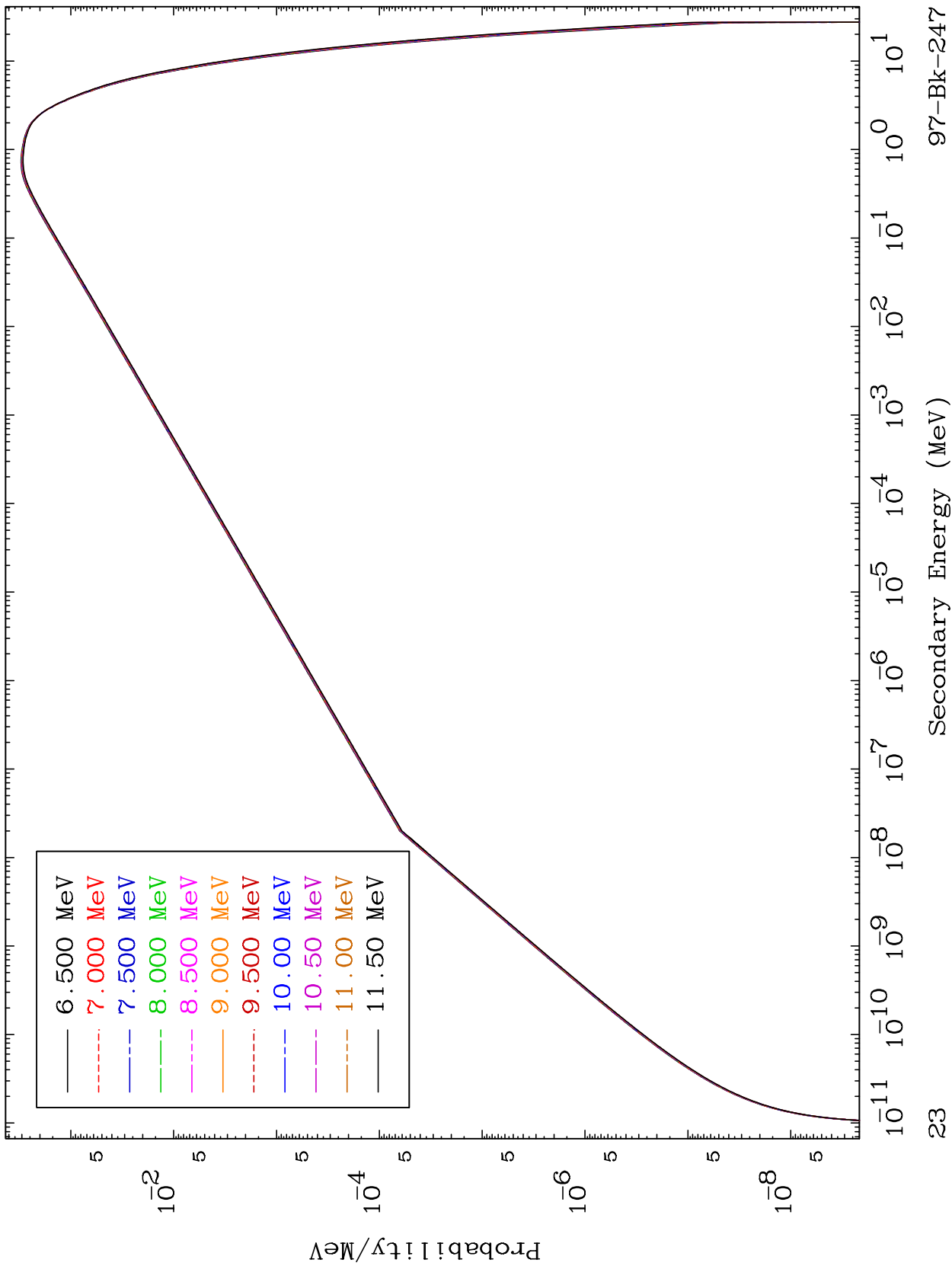
Fission Energy Distributions



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Fission Energy Distributions

97-Bk-247



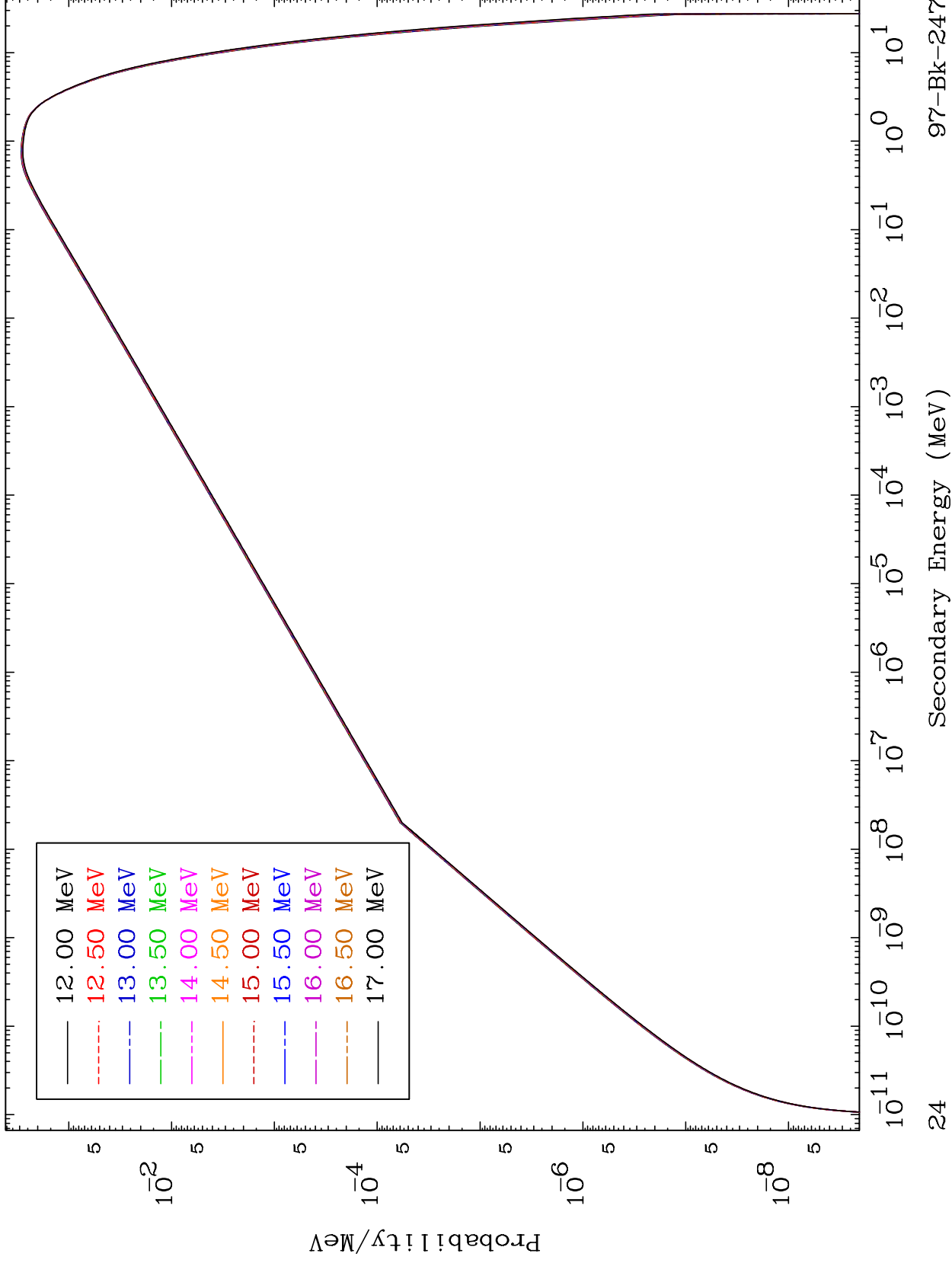
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Secondary Energy (MeV)

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Fission Energy Distributions

97-Bk-247

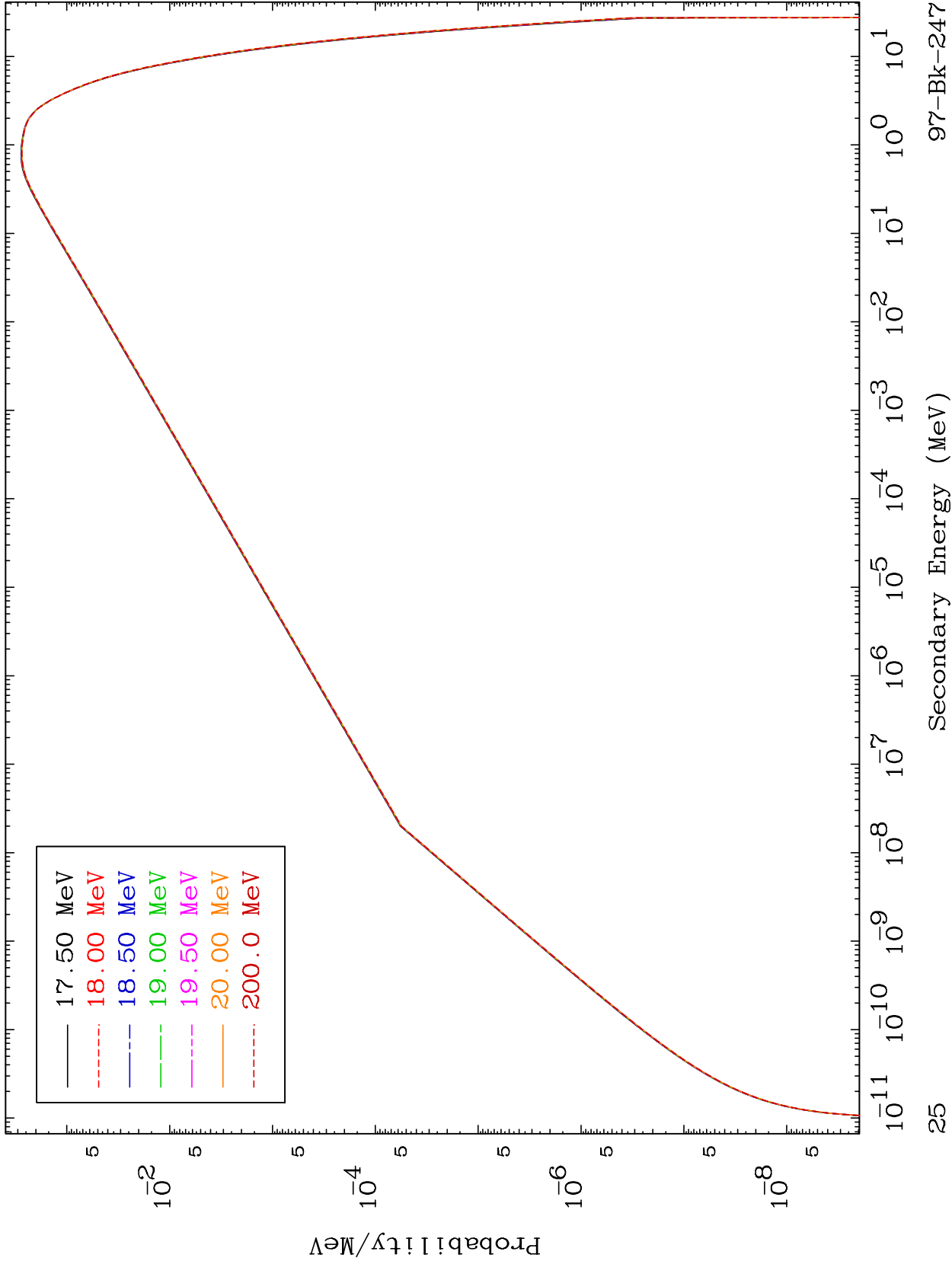


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Fission Energy Distributions

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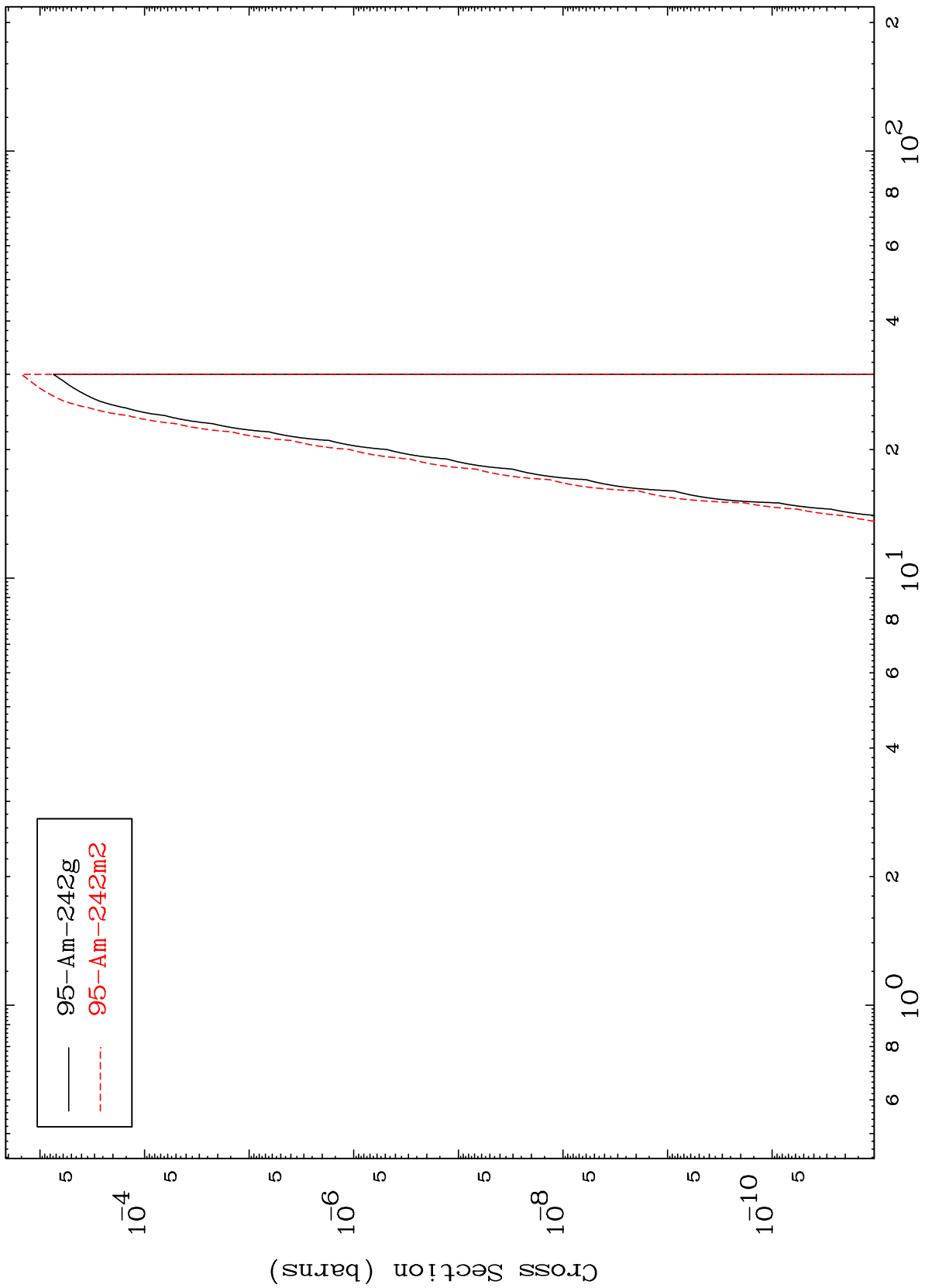


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(n,2n) α

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

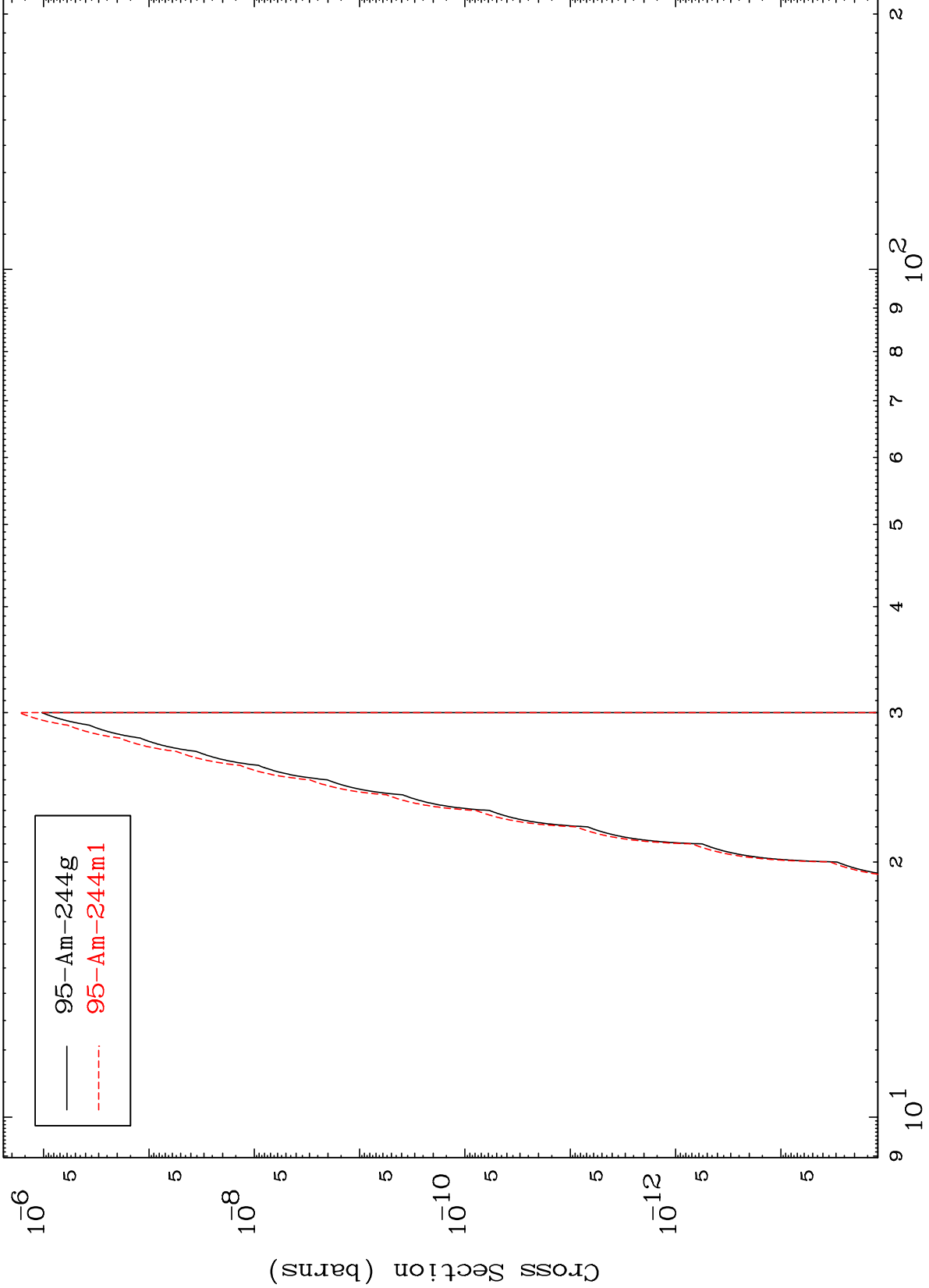
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(n,n') He-3

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Radionuclide Production Cross Section



95-Am-244g
95-Am-244m1

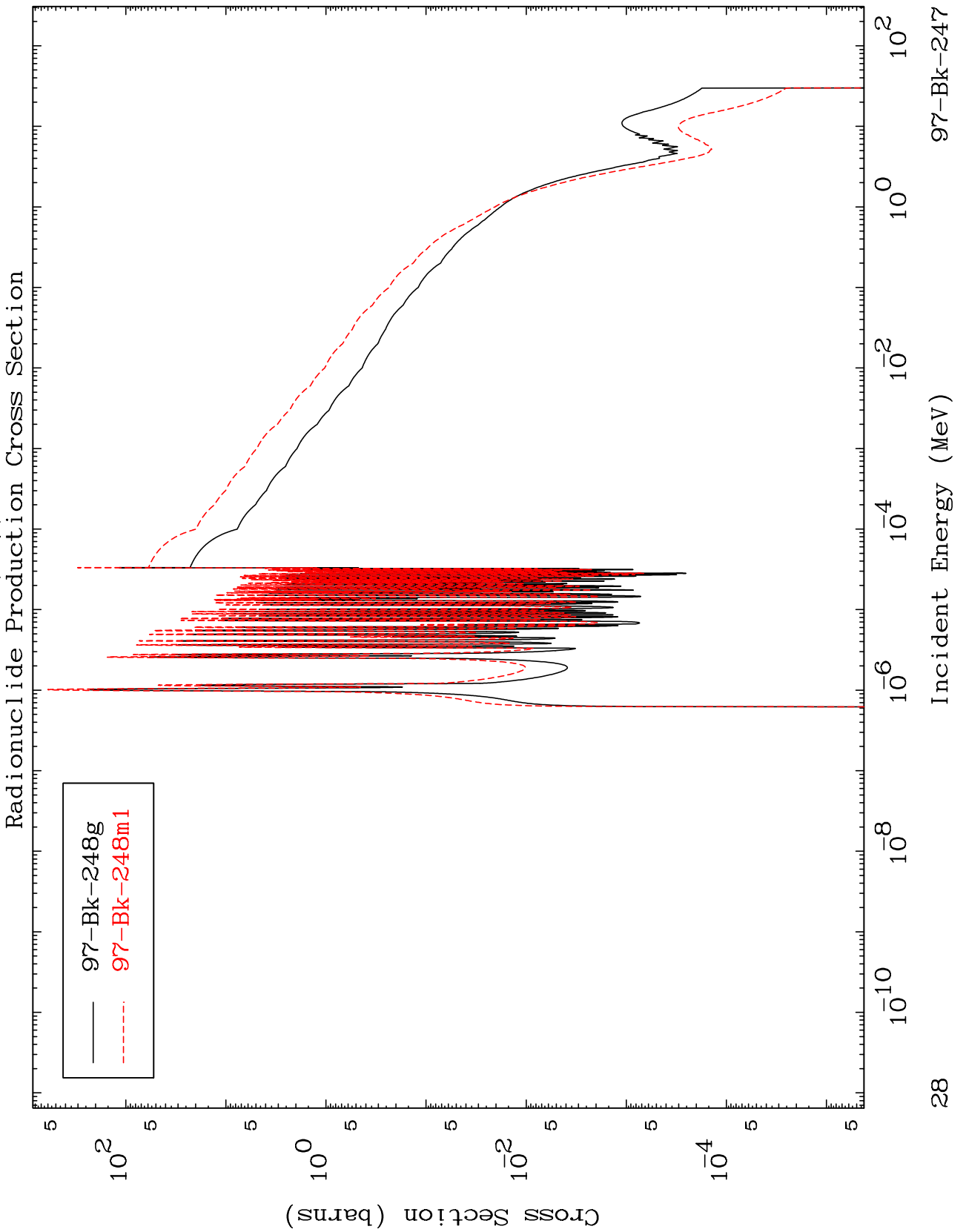
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Incident Energy (MeV)

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97-Bk-247

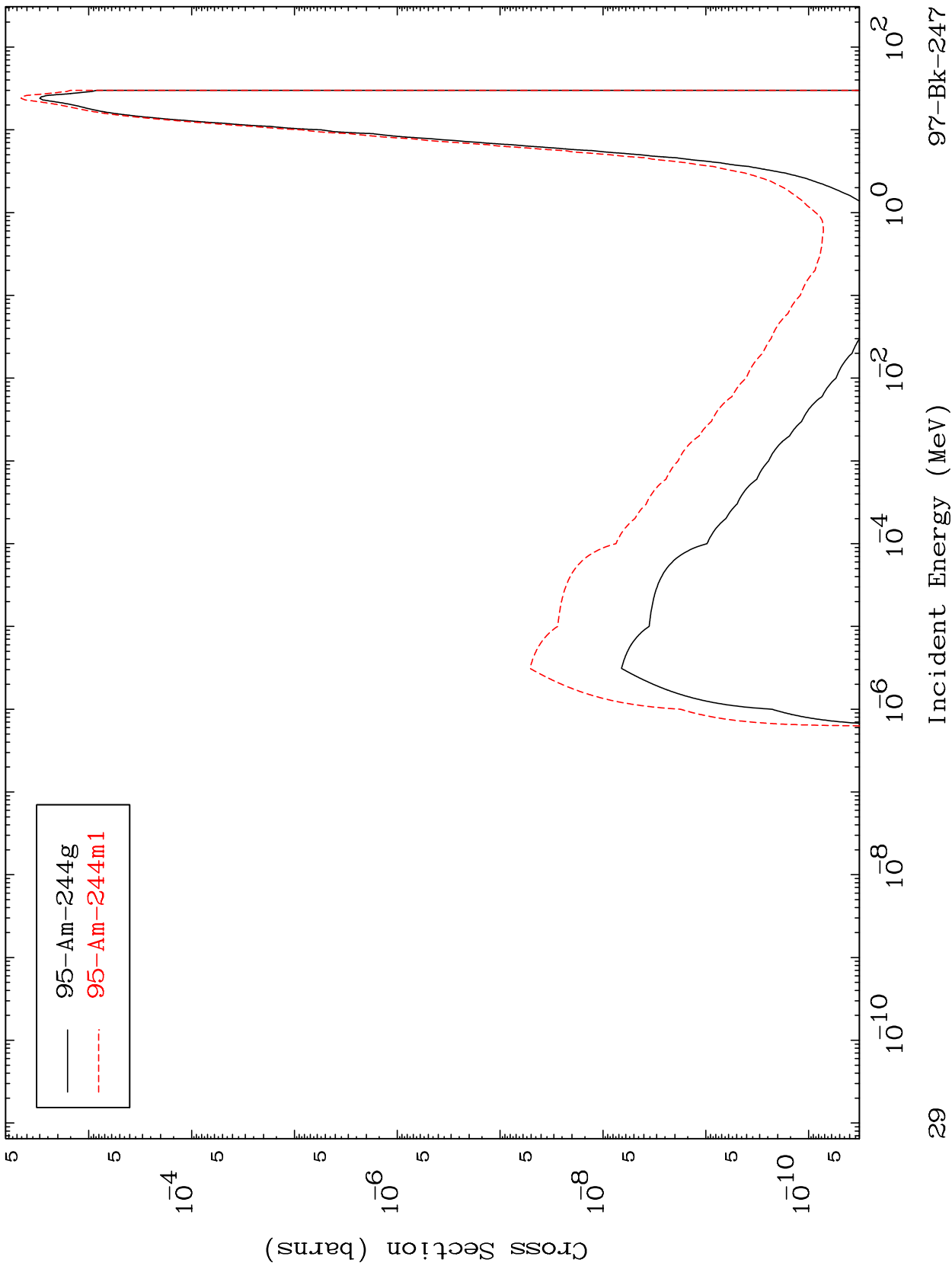


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(n, α)

97-Bk-247

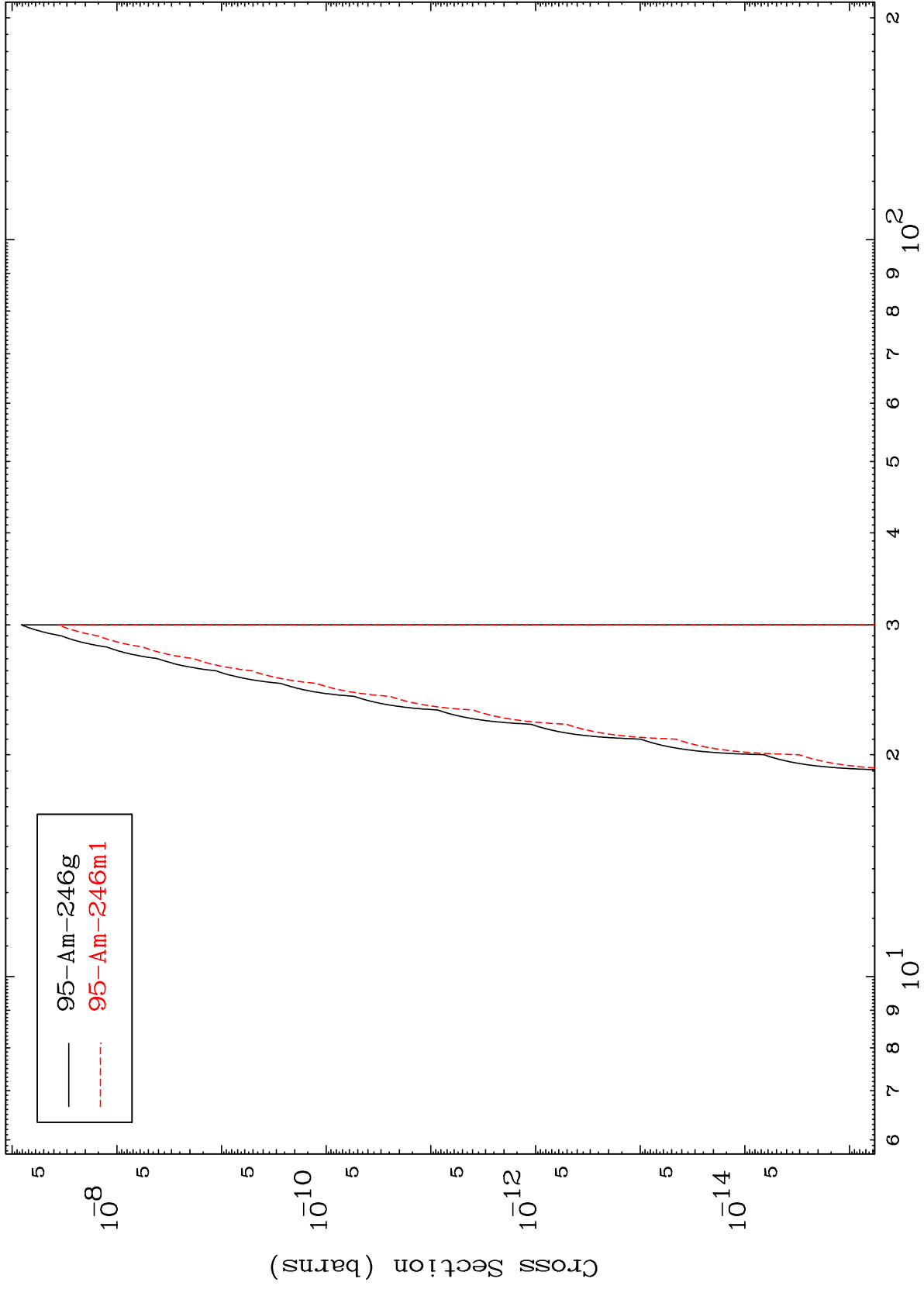
Radionuclide Production Cross Section



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97-Bk-247

Radionuclide Production Cross Section
(n,2p)



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Incident Energy (MeV)

97-Bk-247