

Program EVALPLOT
(Version 2021-1)

by

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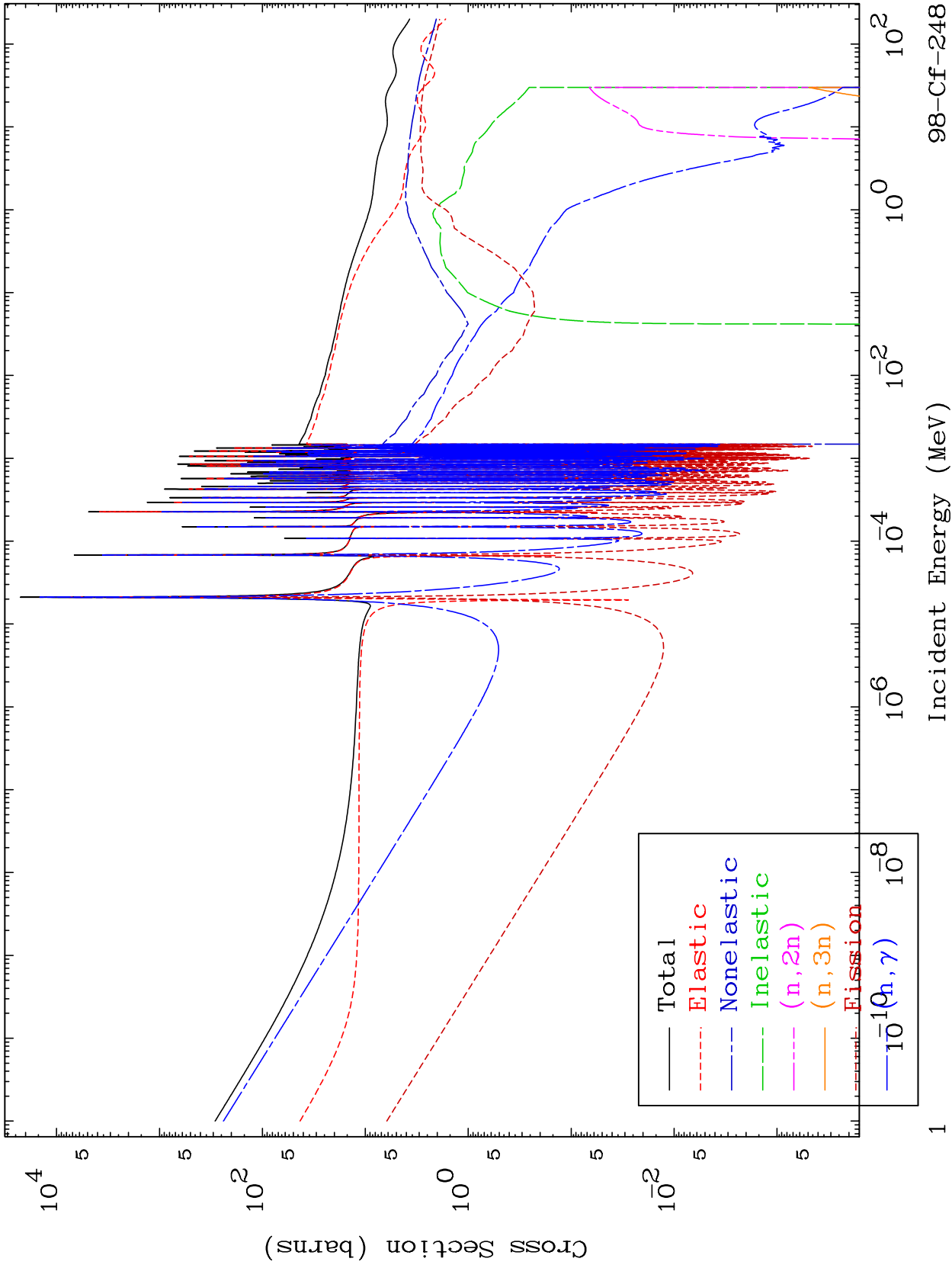
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Press Mouse Button to Start

MAT 9849

Neutron Major
293 Kelvin Cross Sections

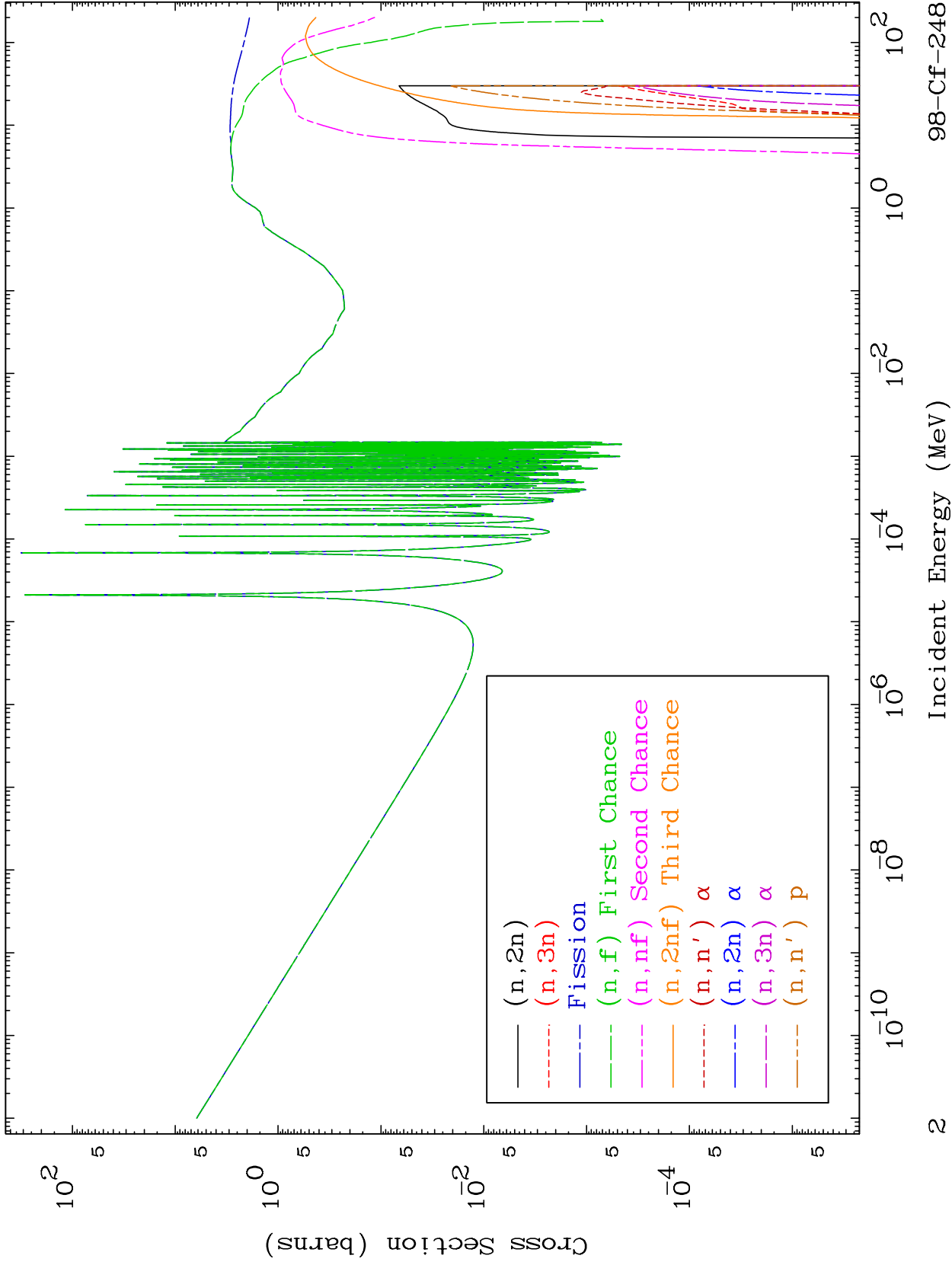
98-Cf-248

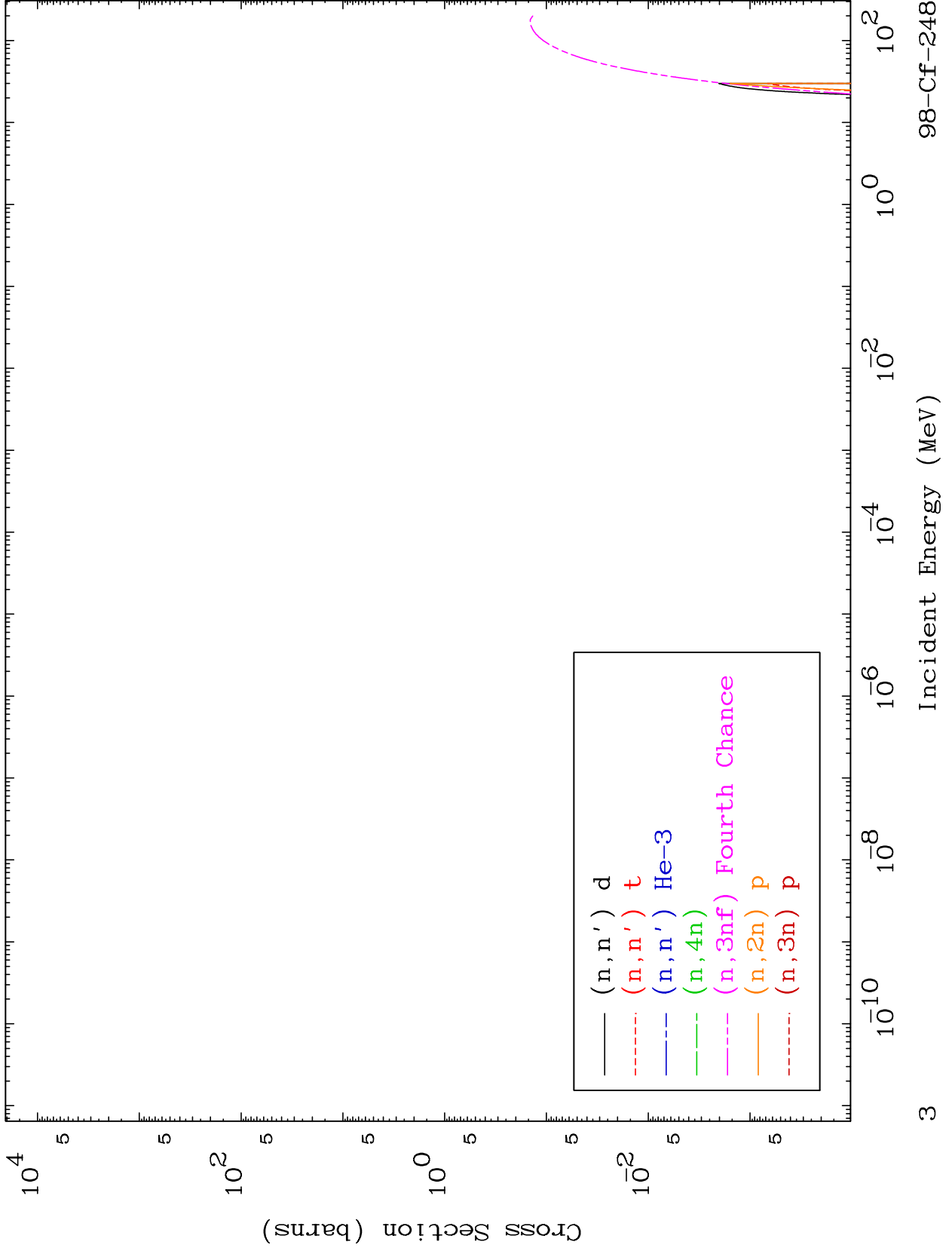


MAT 9849

Neutron Absorption
293 Kelvin Cross Sections

98-Cf-248

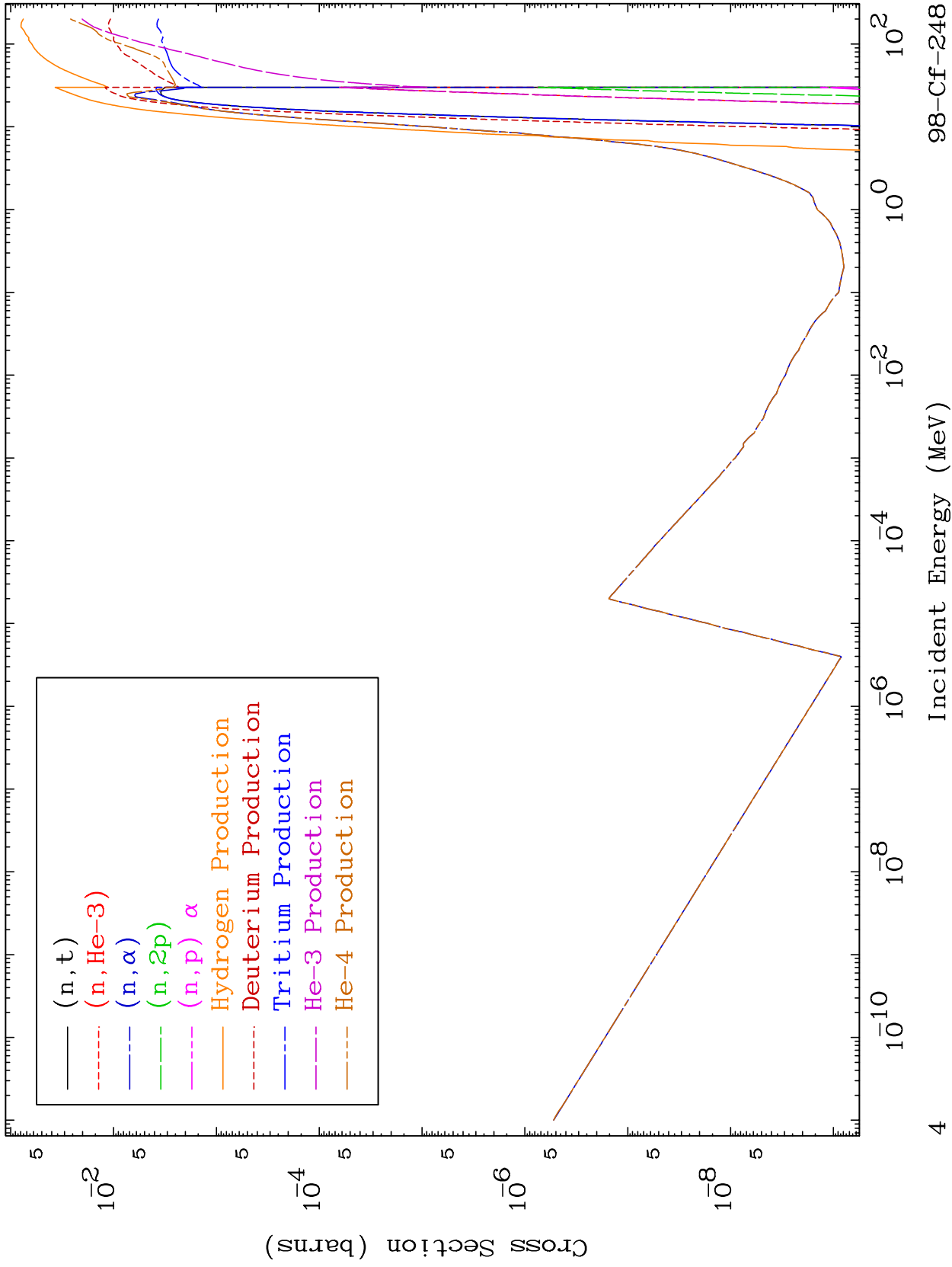




MAT 9849

Neutron Absorption
293 Kelvin Cross Sections

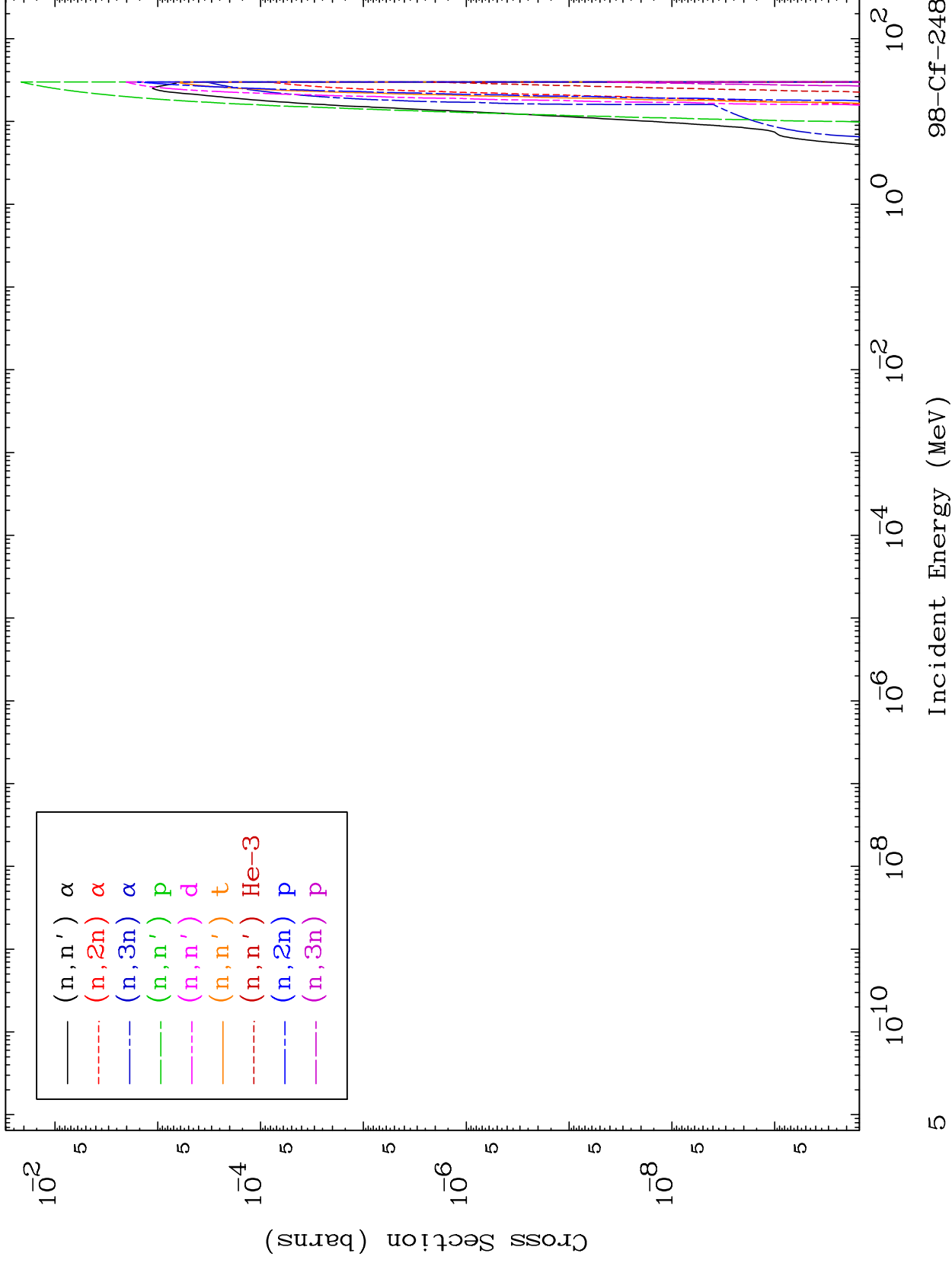
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MAT 9849

Charged Particle
293 Kelvin Cross Sections

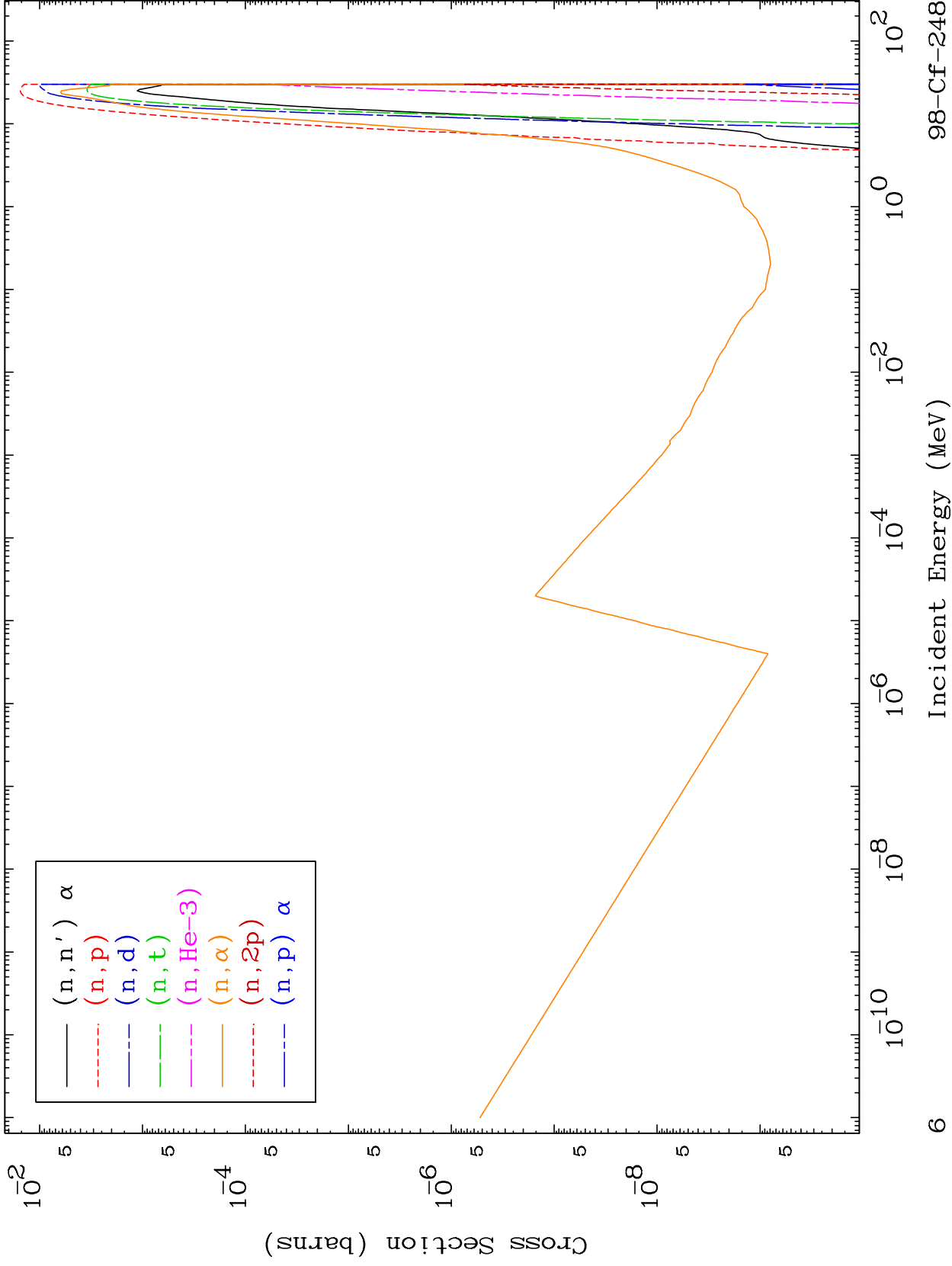
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MAT 9849

Charged Particle
293 Kelvin Cross Sections

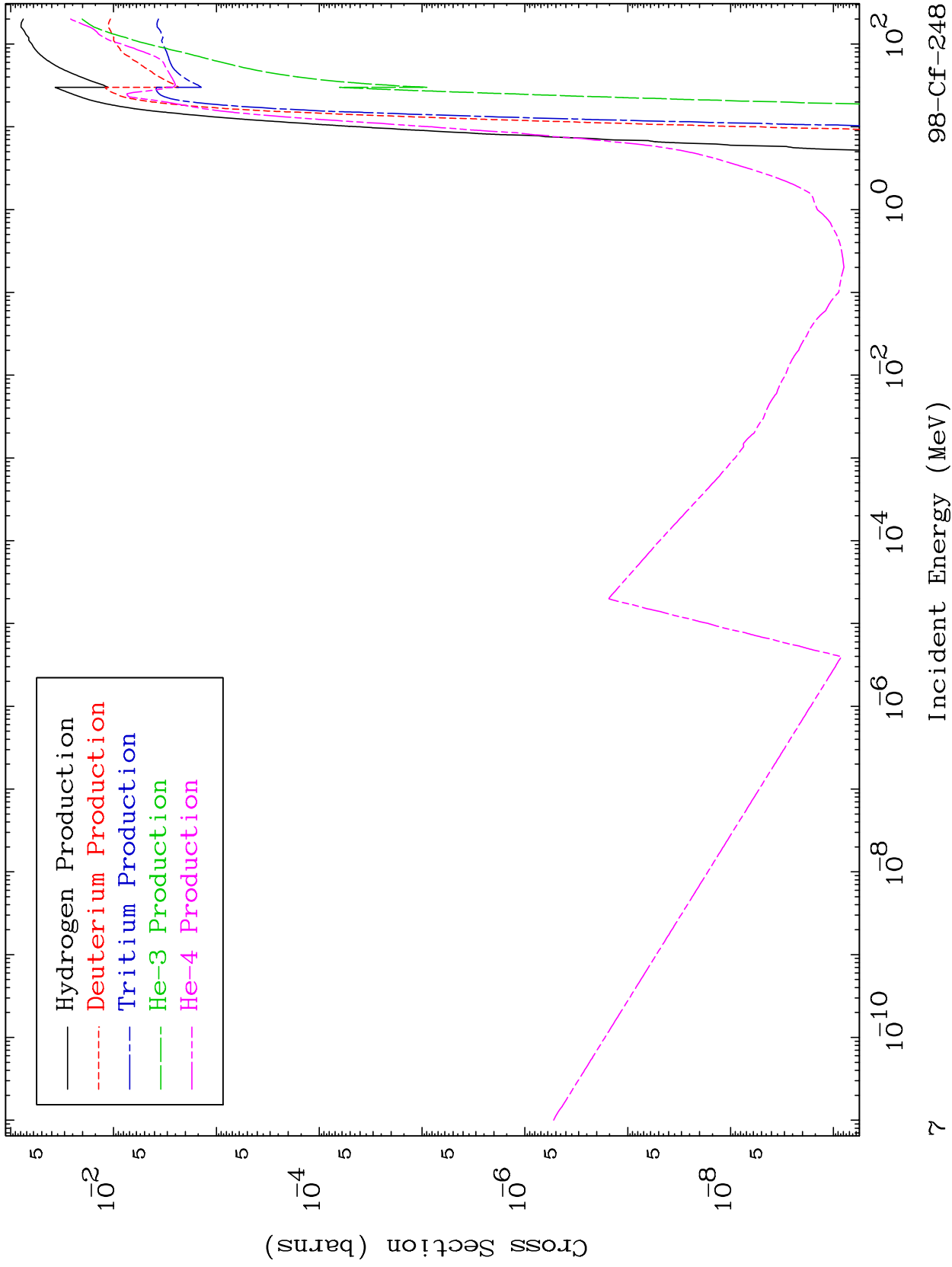
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Particle Production
293 Kelvin Cross Sections

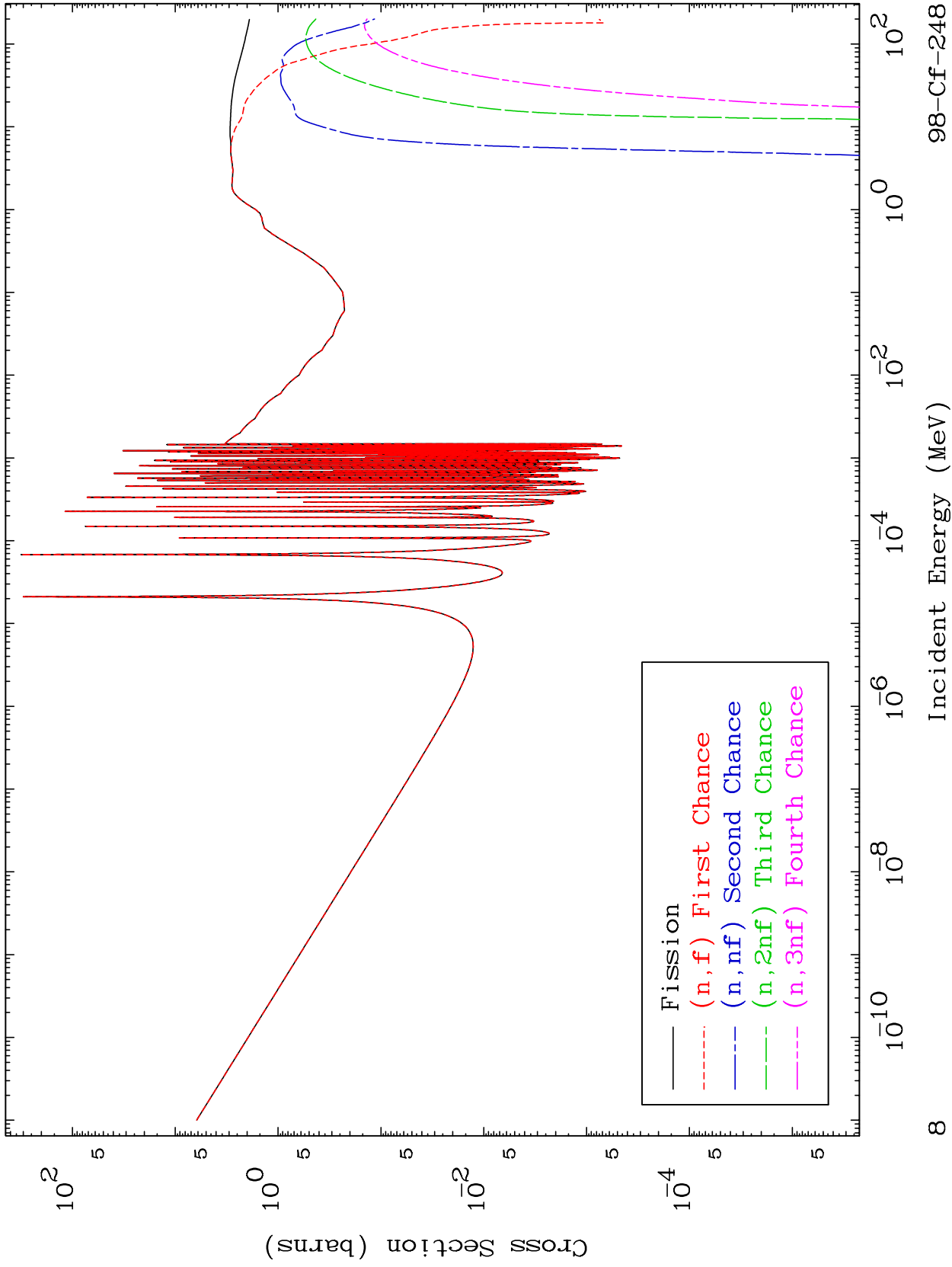
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Fission
293 Kelvin Cross Sections

98-Cf-248

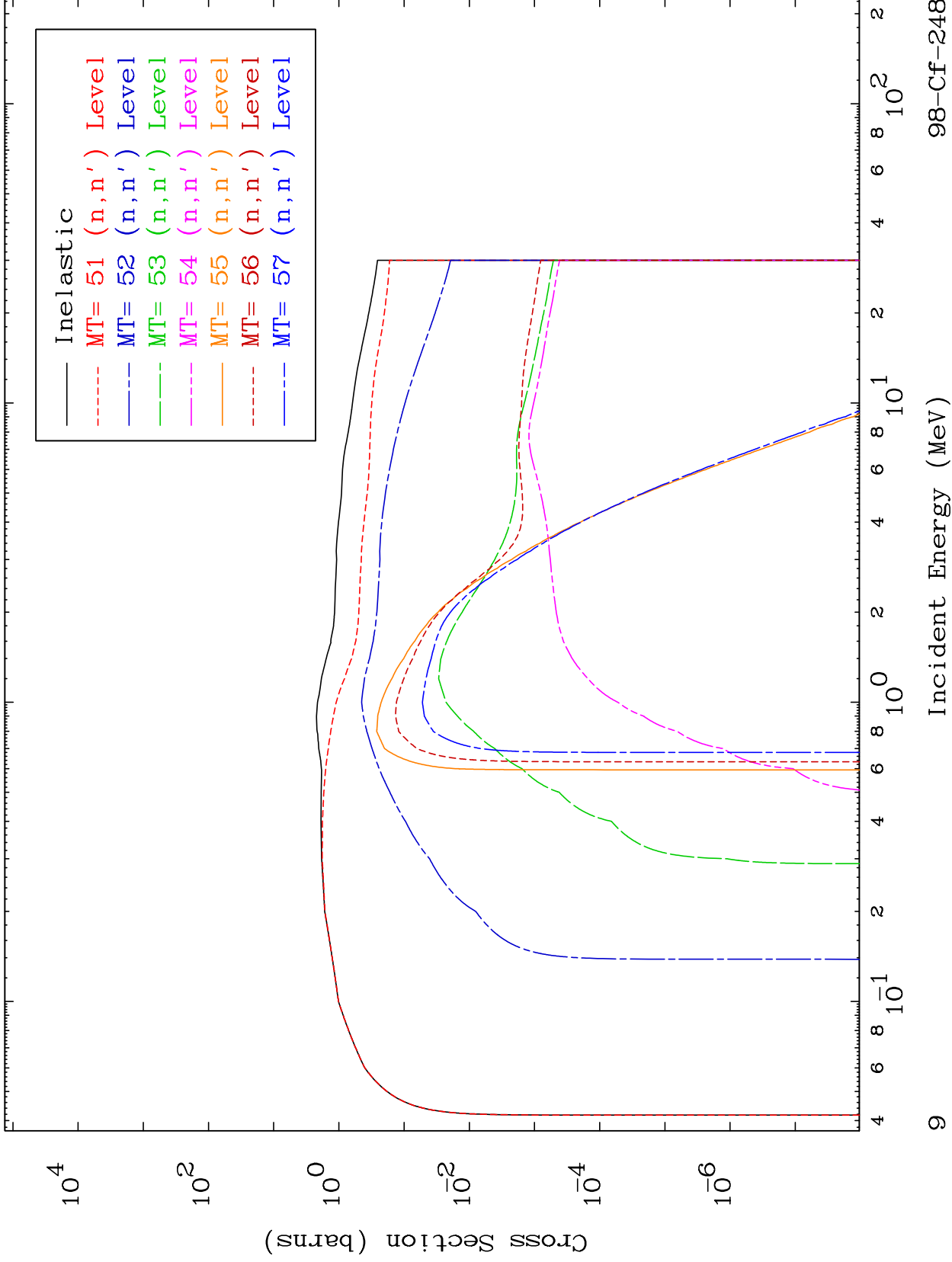


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(n,n') Levels

98-Cf-248

293 Kelvin Cross Sections

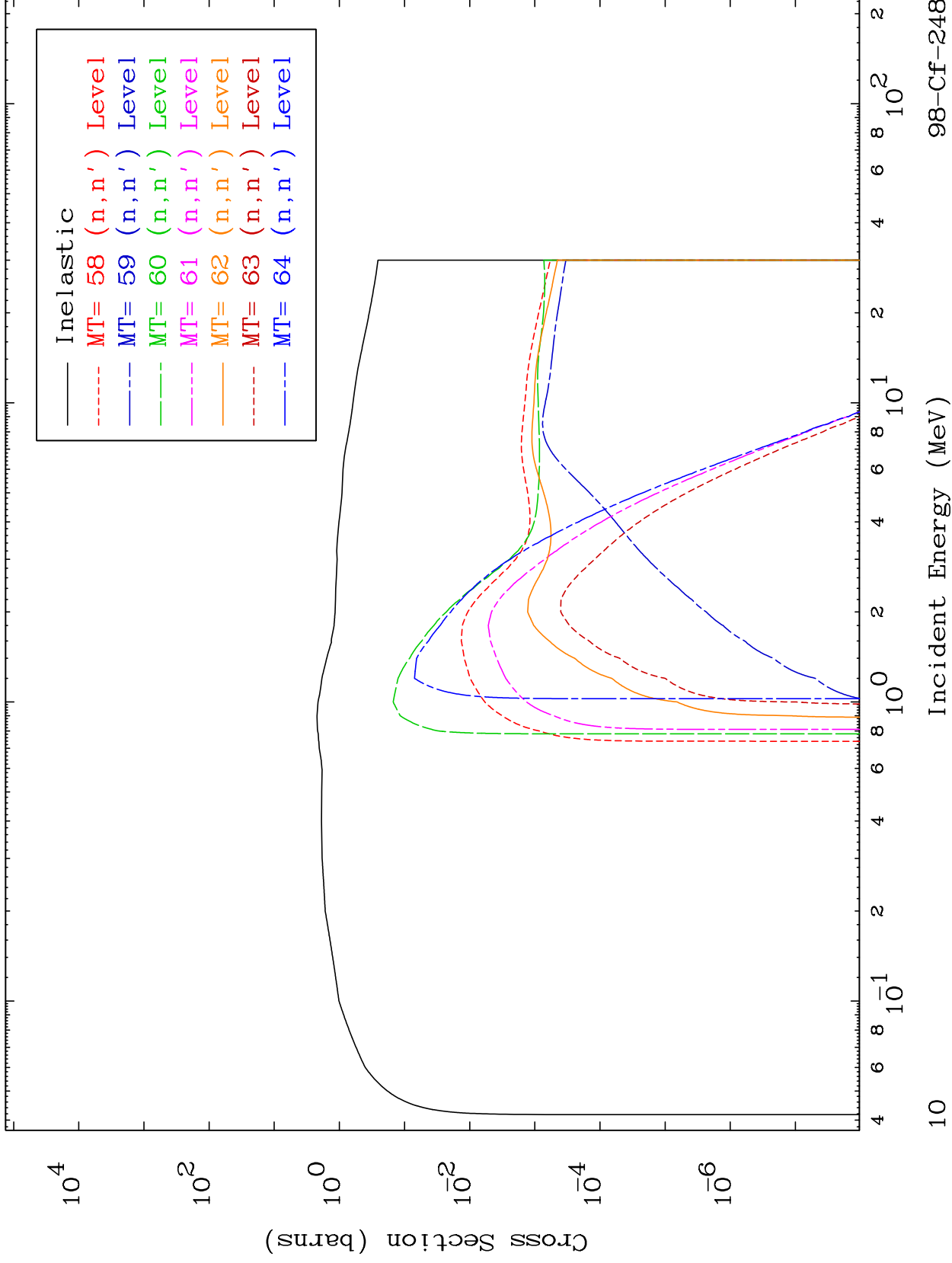


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(n,n') Levels

98-Cf-248

293 Kelvin Cross Sections



10

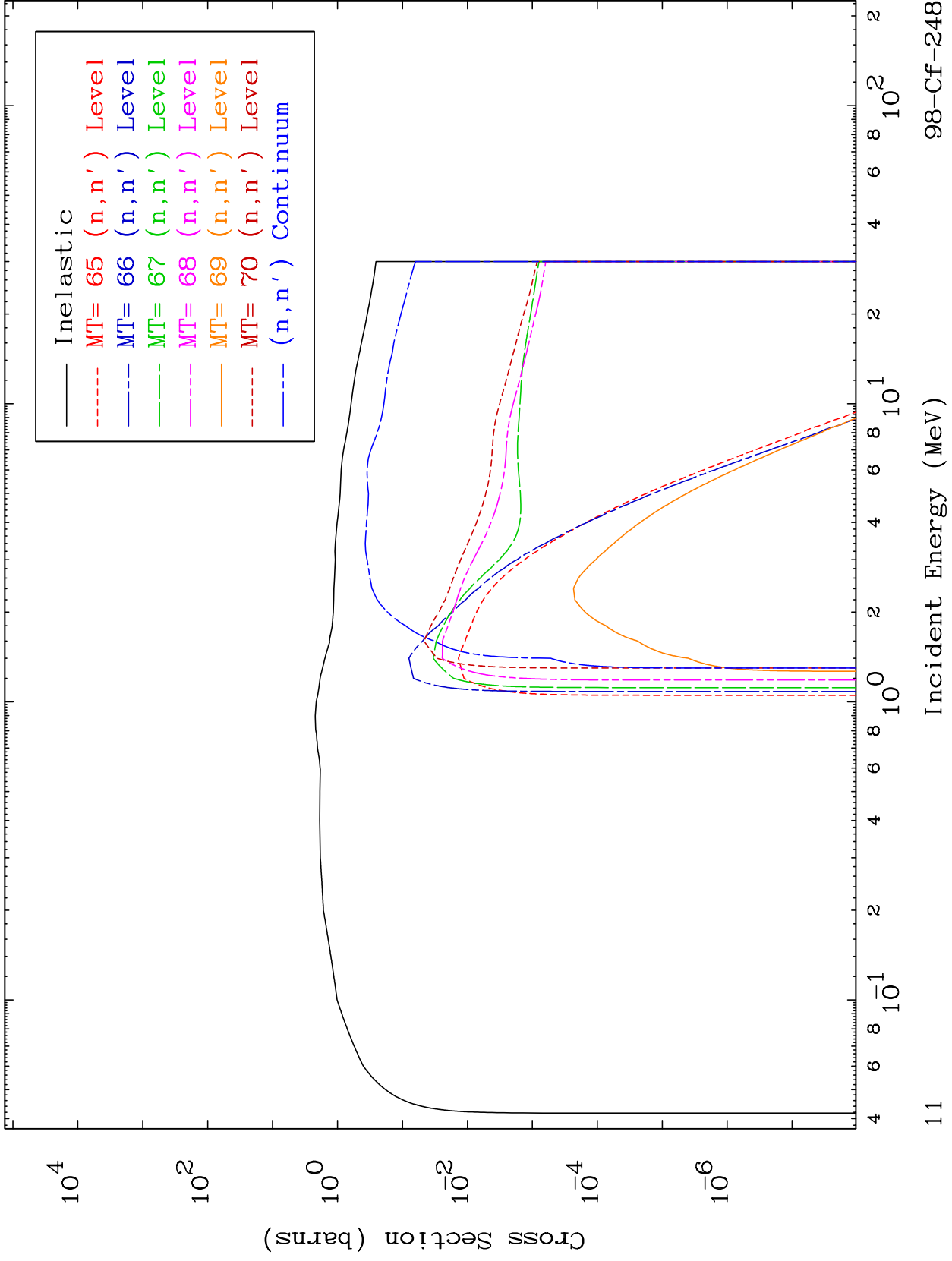
Incident Energy (MeV)

98-Cf-248

MAT 9849

(n,n') Levels
293 Kelvin Cross Sections

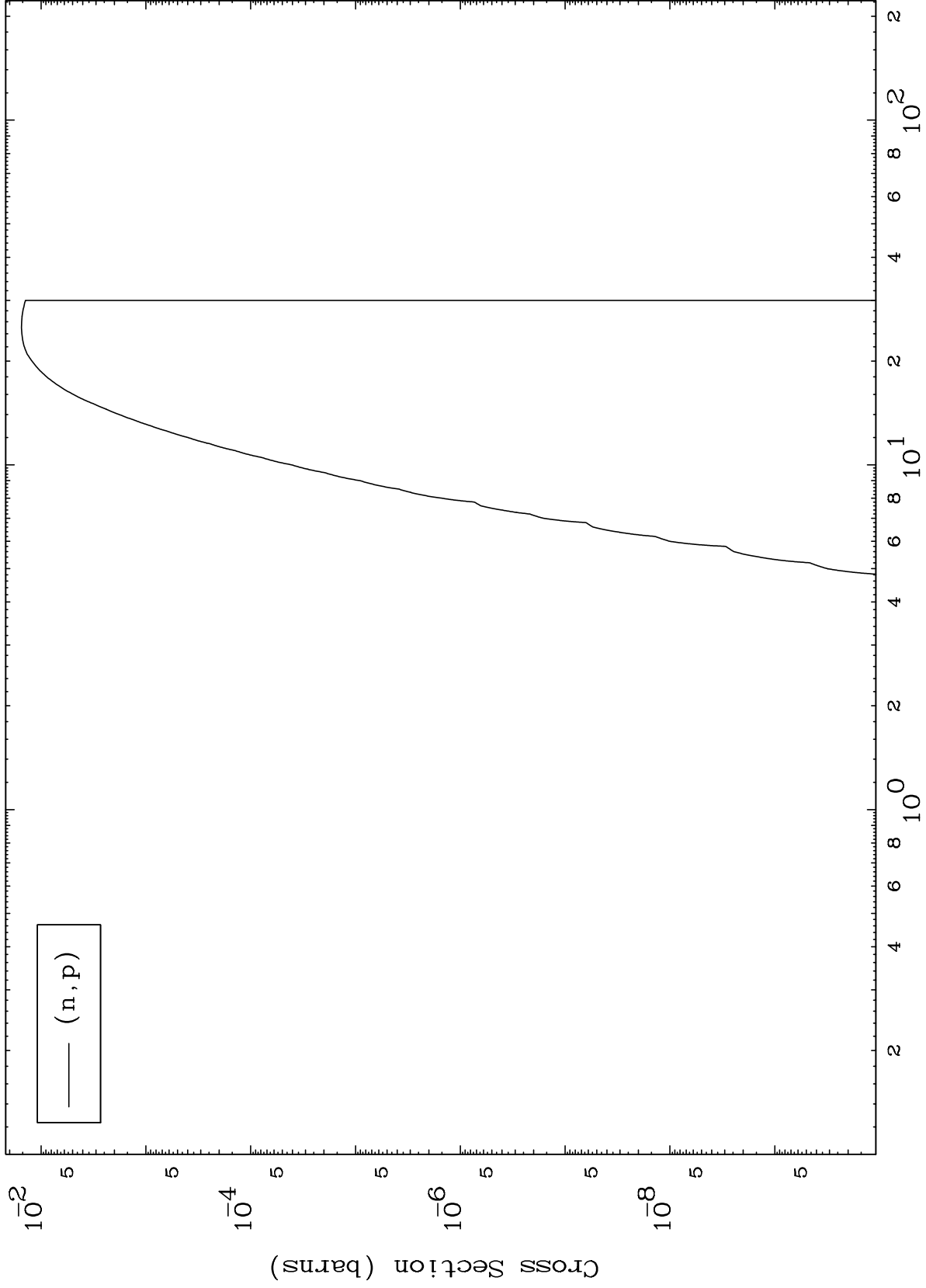
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(n,p) Levels
293 Kelvin Cross Sections

98-Cf-248



12

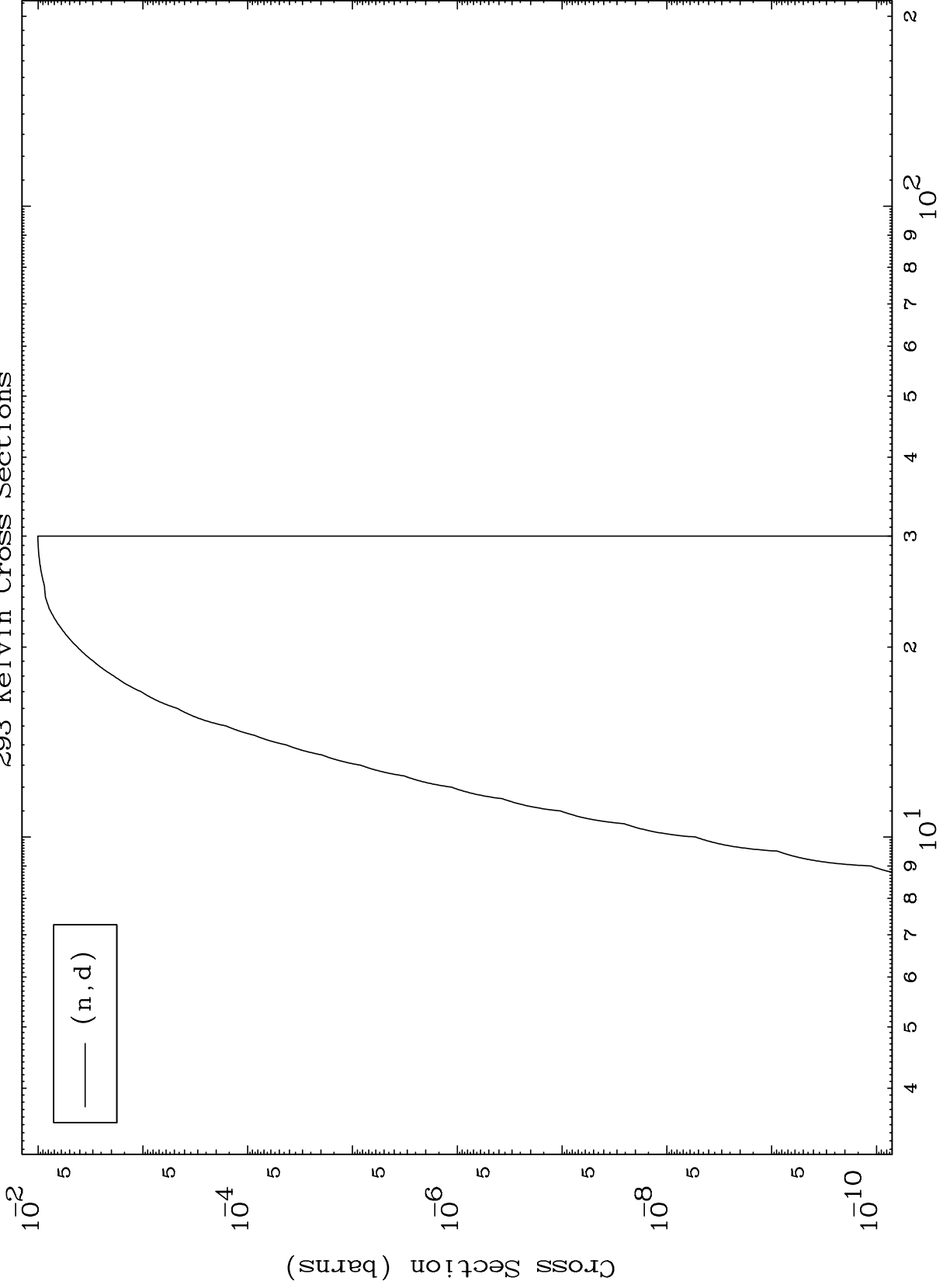
Incident Energy (MeV)

98-Cf-248

MAT 9849

(n,d) Levels
293 Kelvin Cross Sections

98-Cf-248



13

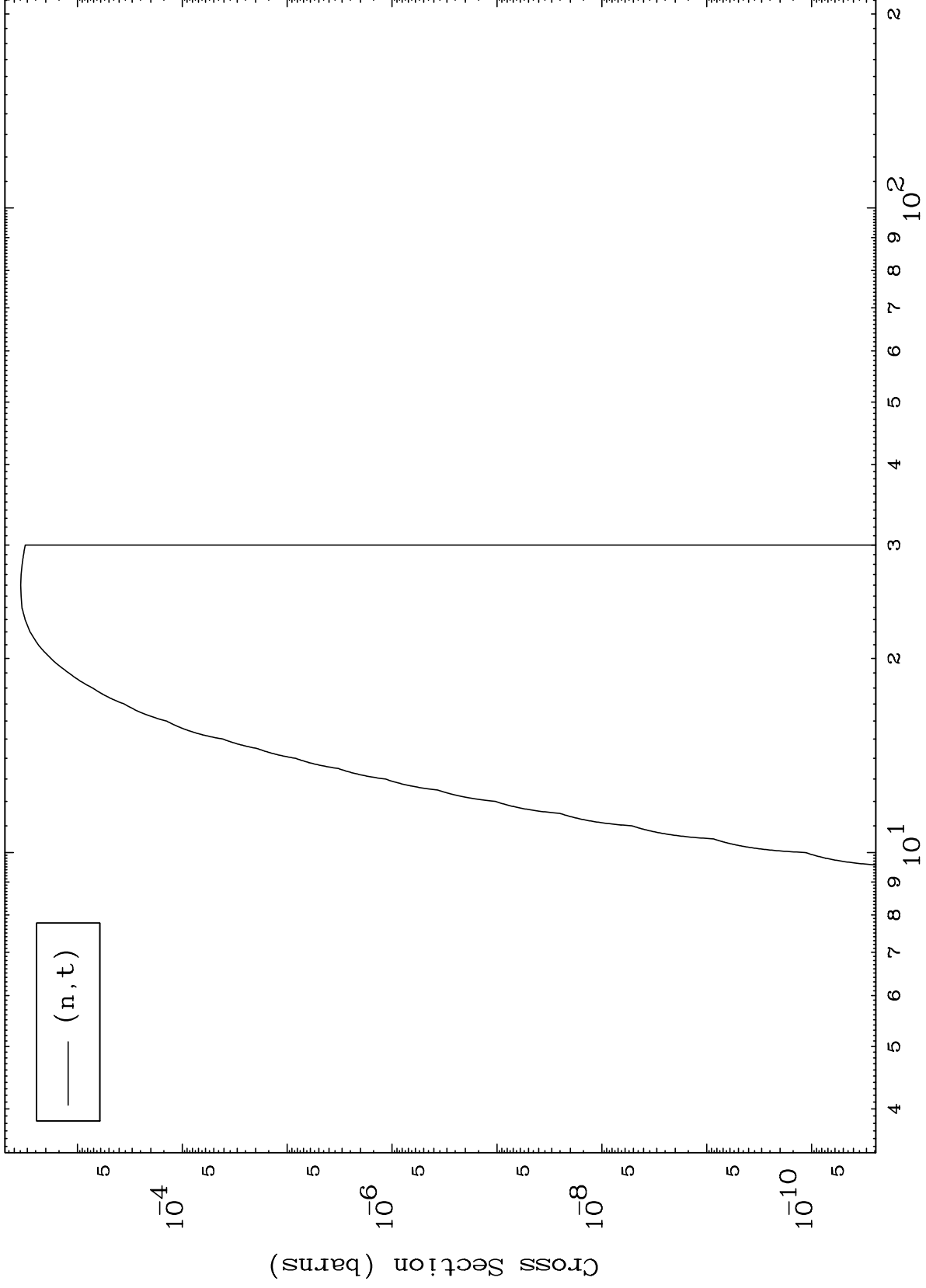
Incident Energy (MeV)

98-Cf-248

MAT 9849

(n,t) Levels
293 Kelvin Cross Sections

98-Cf-248



14

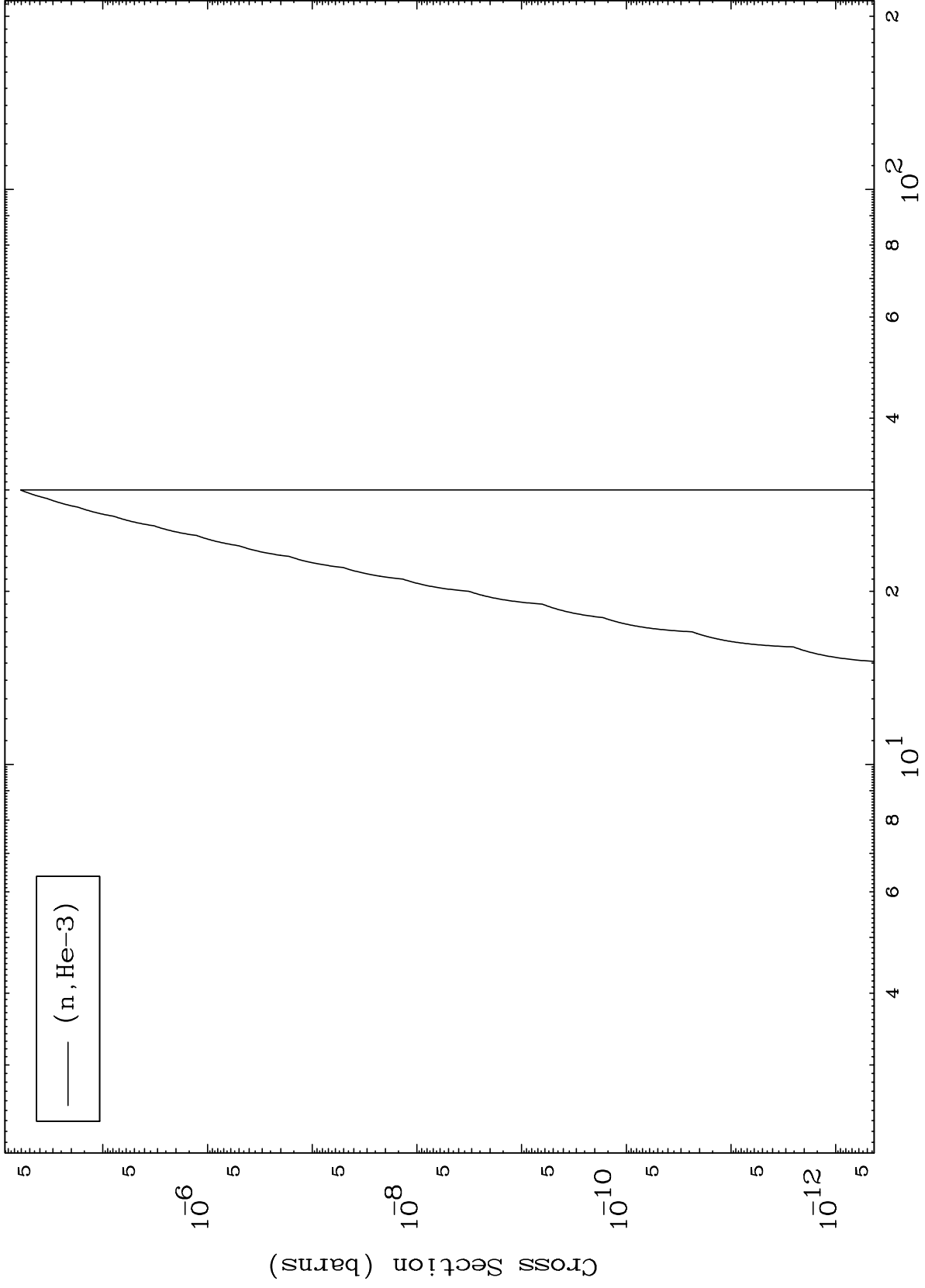
Incident Energy (MeV)

98-Cf-248

MAT 9849

(n,He3) Levels
293 Kelvin Cross Sections

98-Cf-248



15

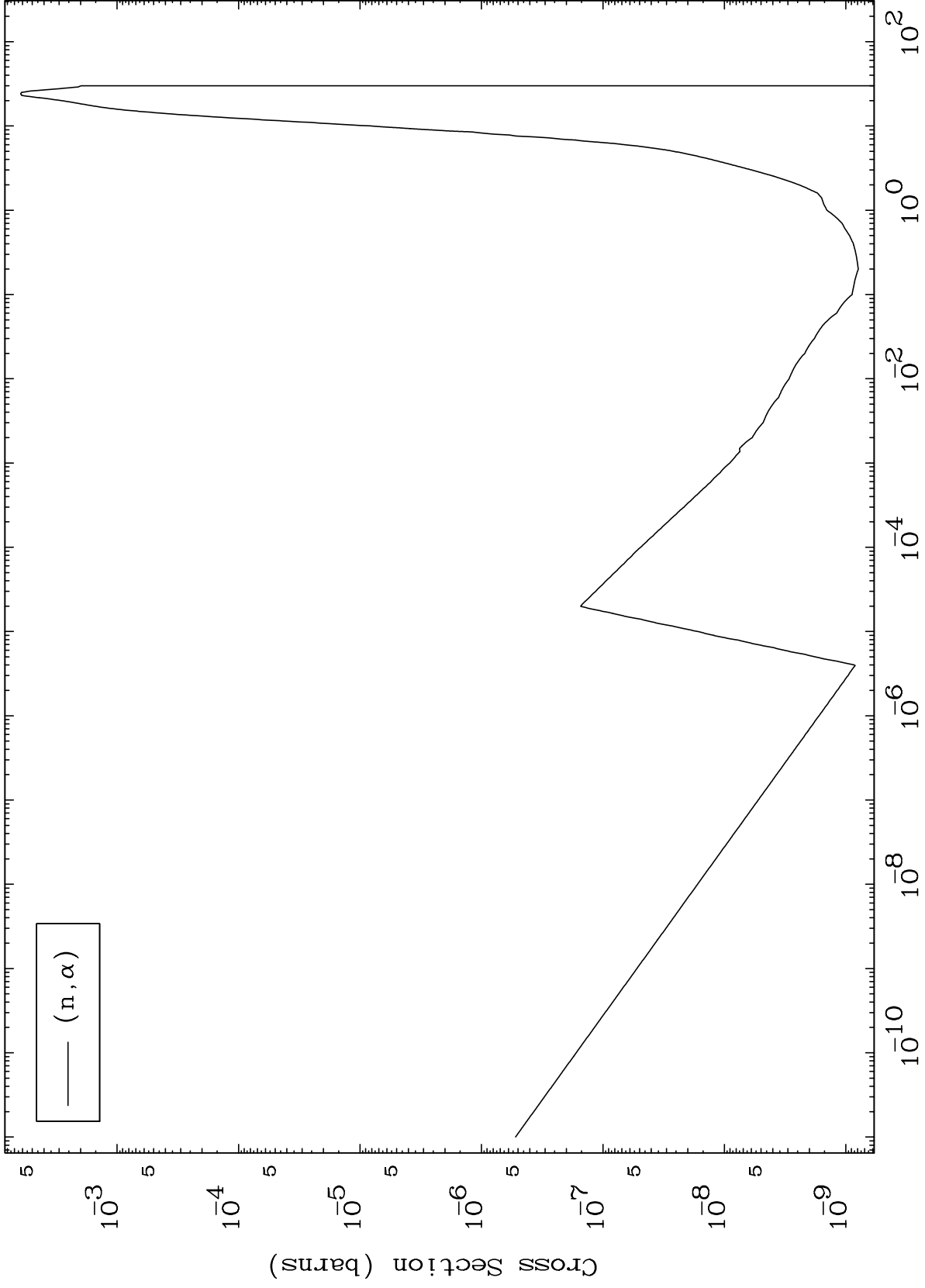
Incident Energy (MeV)

98-Cf-248

MAT 9849

(n, α) Levels
293 Kelvin Cross Sections

98-Cf-248



16

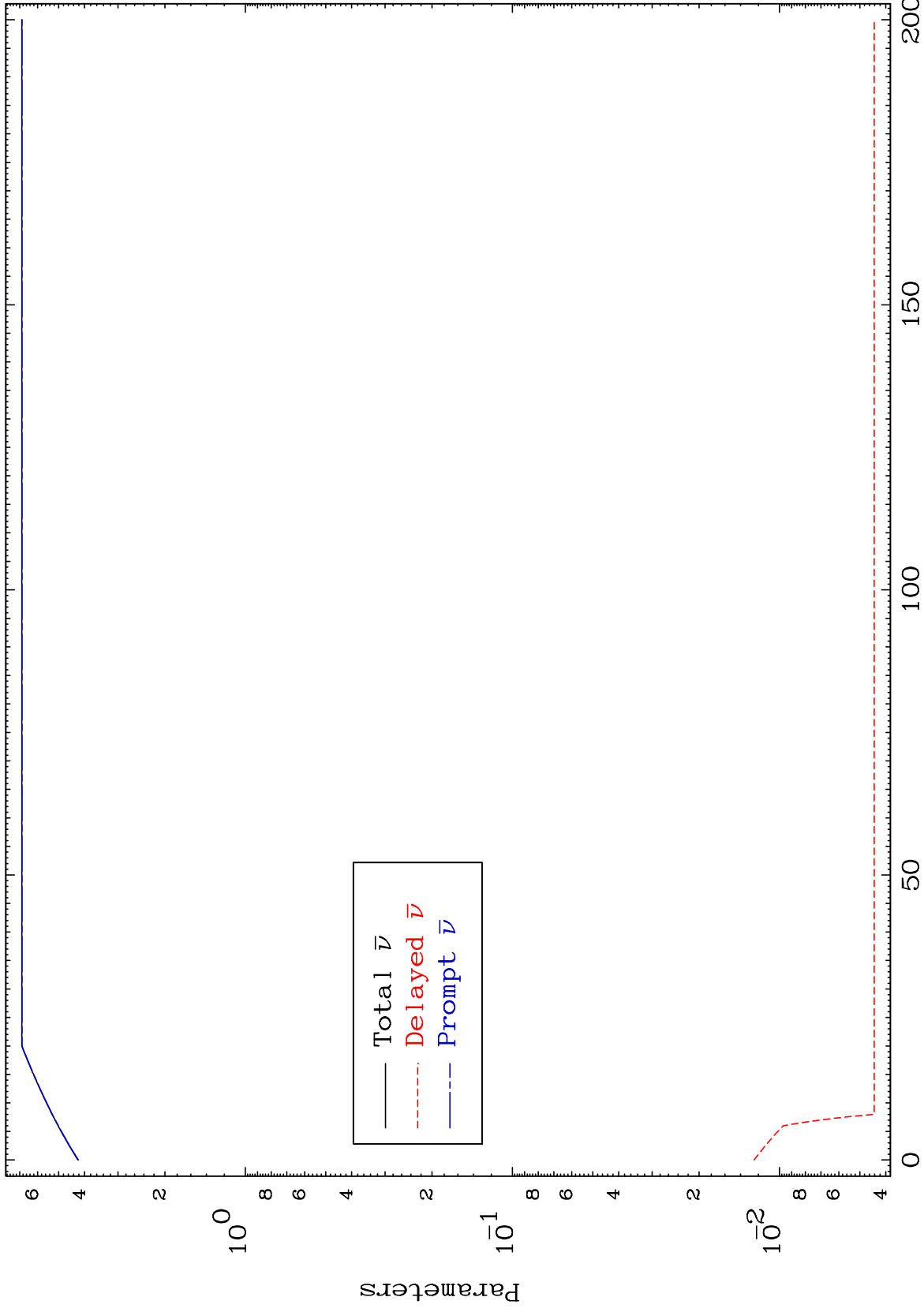
Incident Energy (MeV)

98-Cf-248

MAT 9849

Energy Release
Parameters

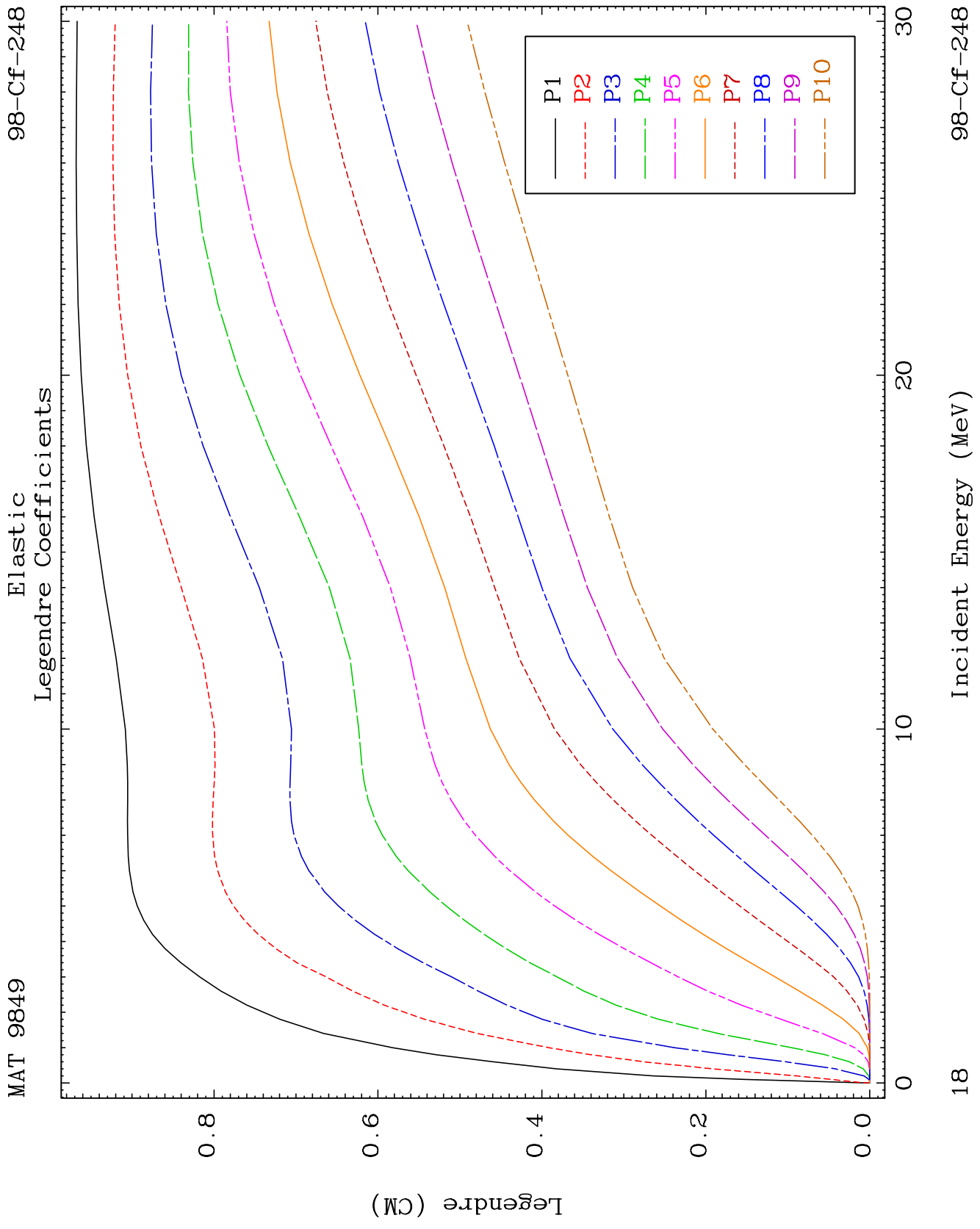
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Incident Energy (MeV)

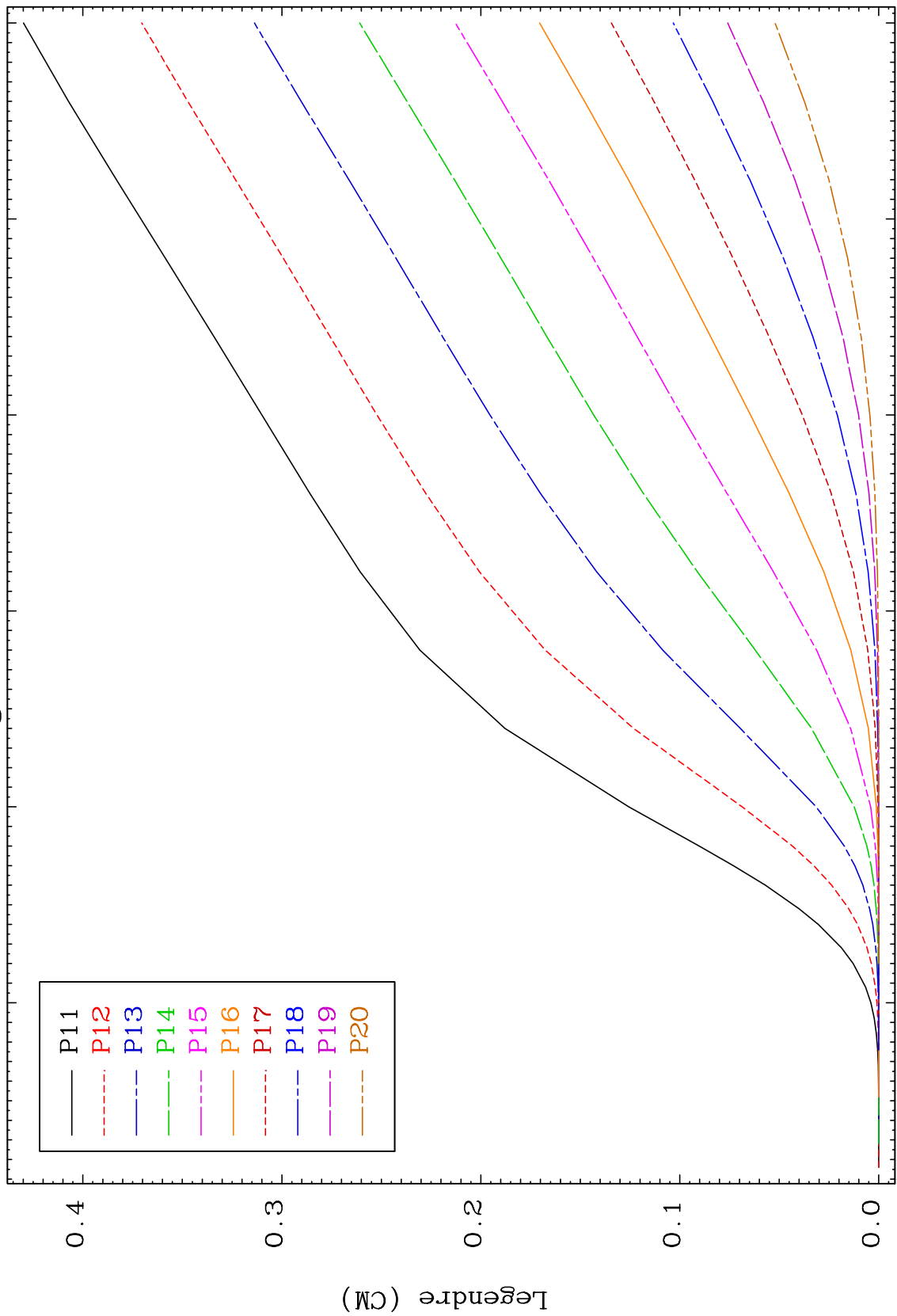
98-Cf-248



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Elastic Legendre Coefficients

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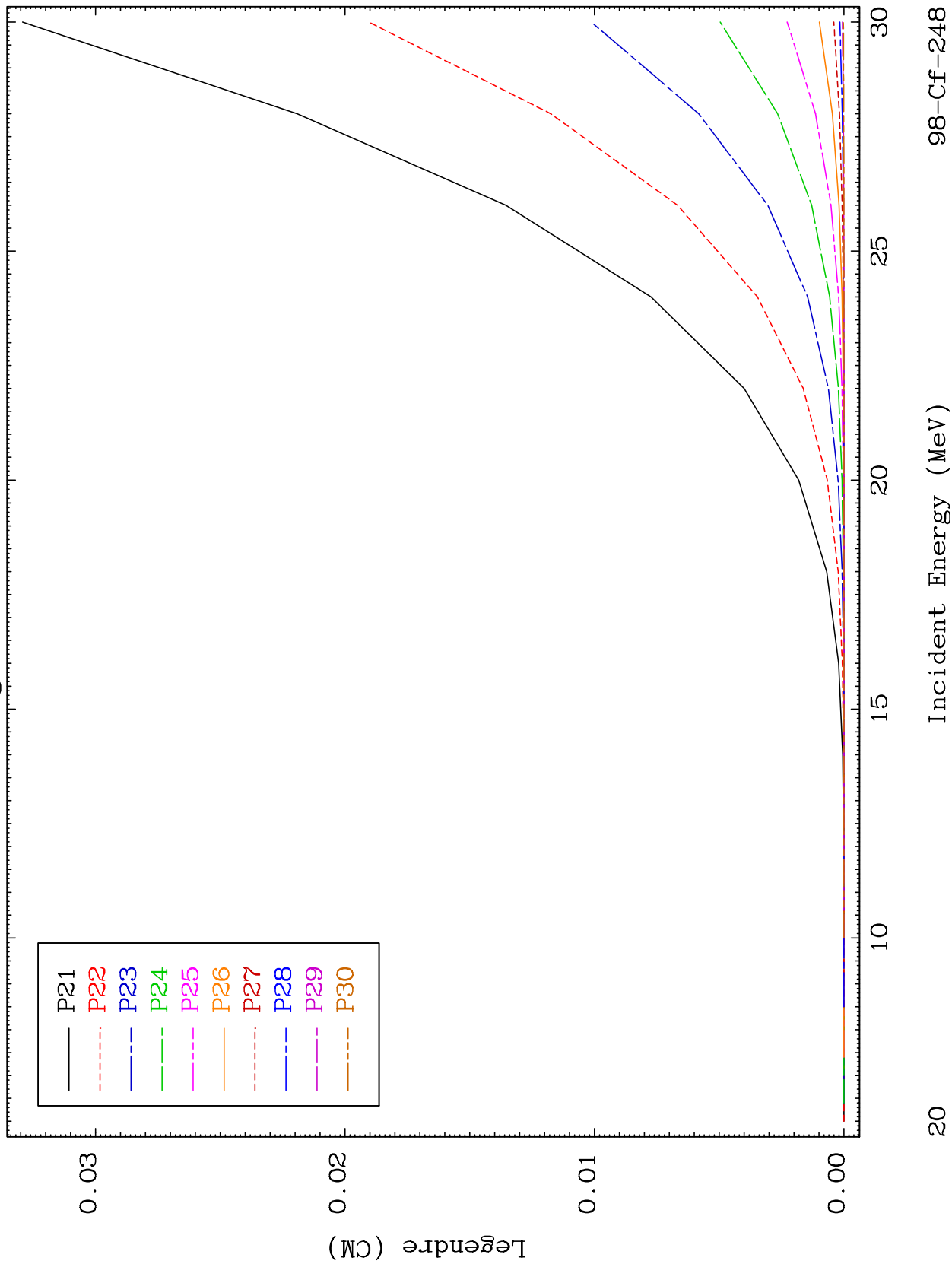
Incident Energy (MeV)

98-Cf-248

MAT 9849

Elastic Legendre Coefficients

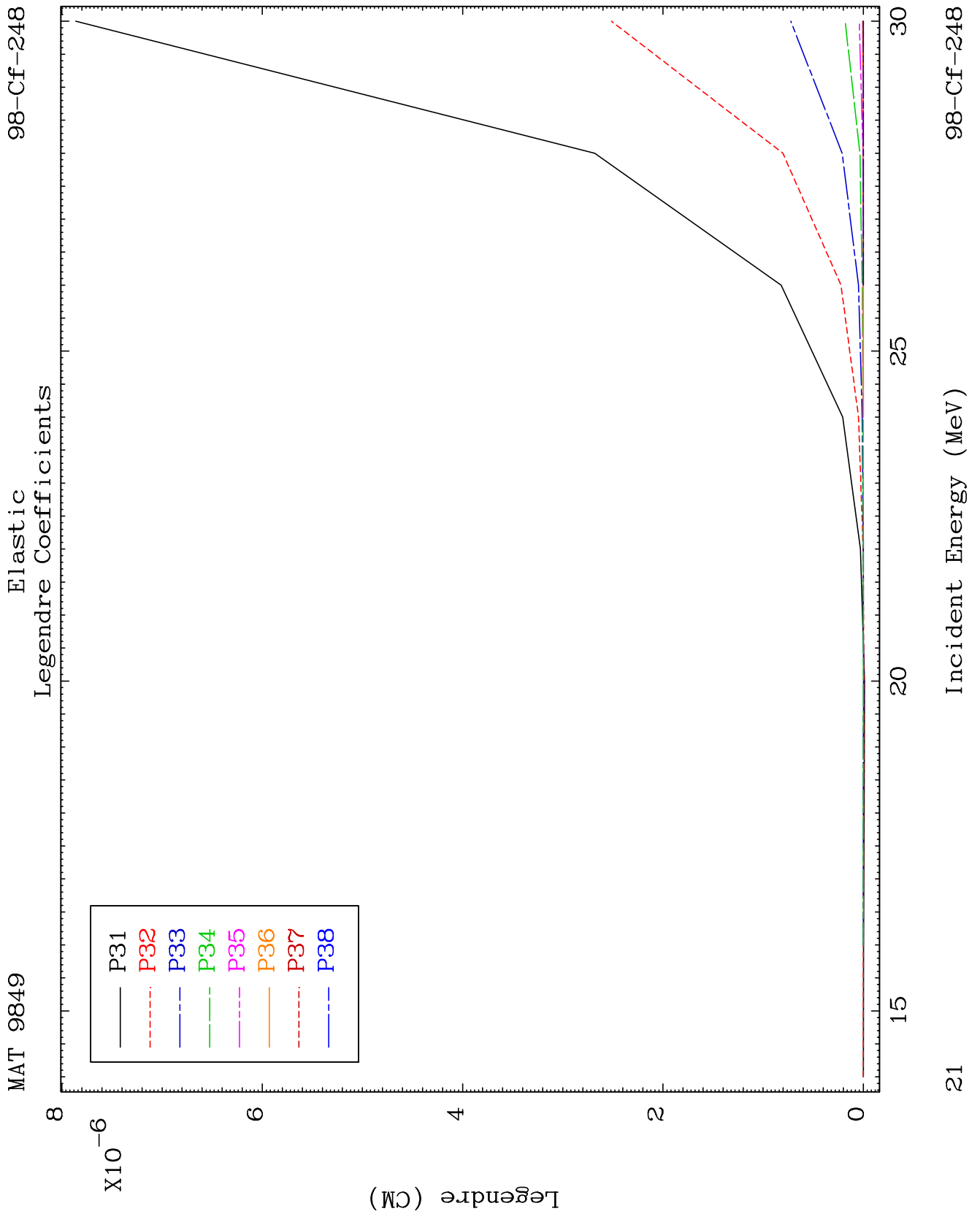
98-Cf-248



98-Cf-248

Incident Energy (MeV)

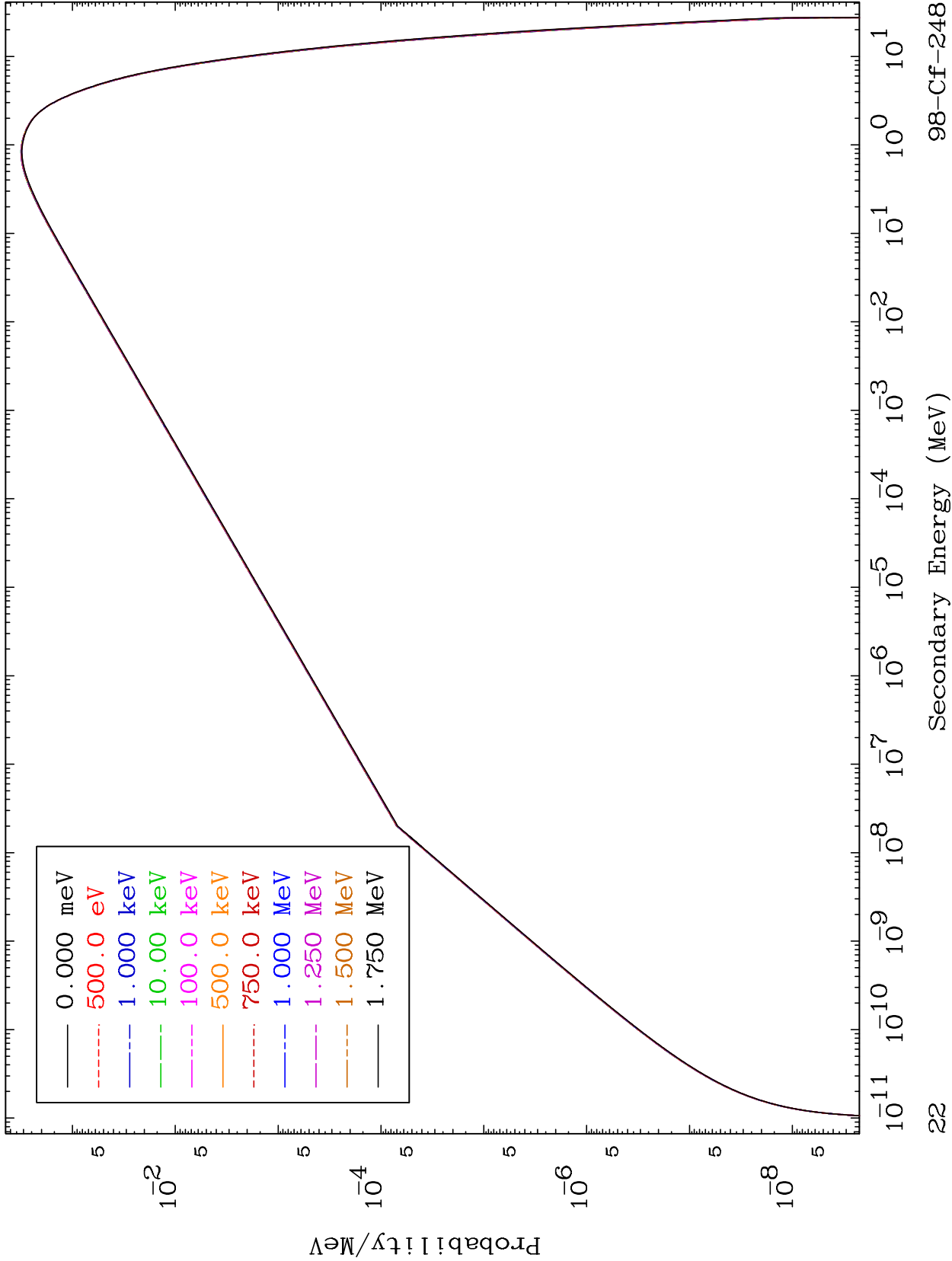
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Fission Energy Distributions

98-Cf-248



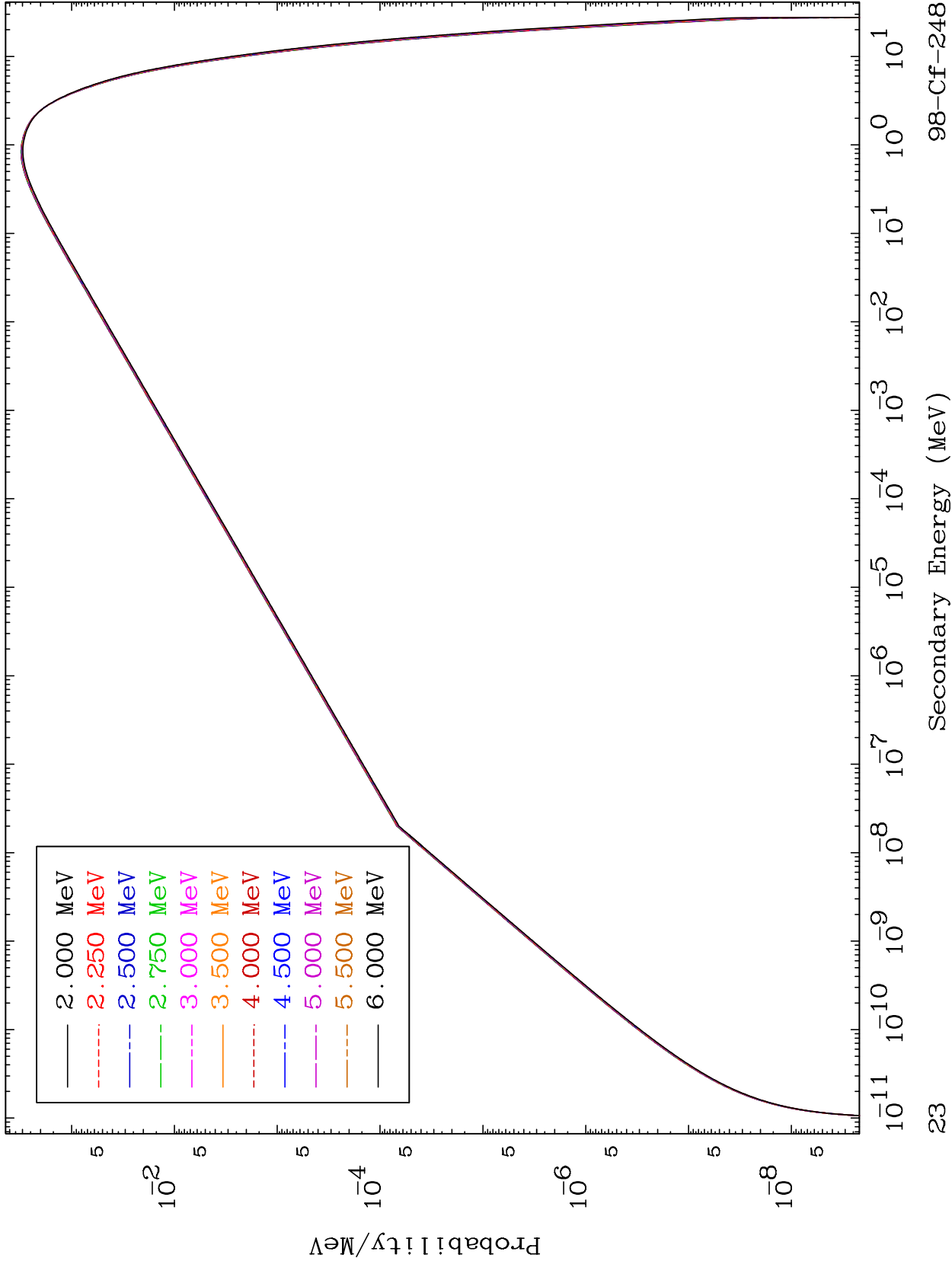
98-Cf-248

Secondary Energy (MeV)

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Fission Energy Distributions

98-Cf-248



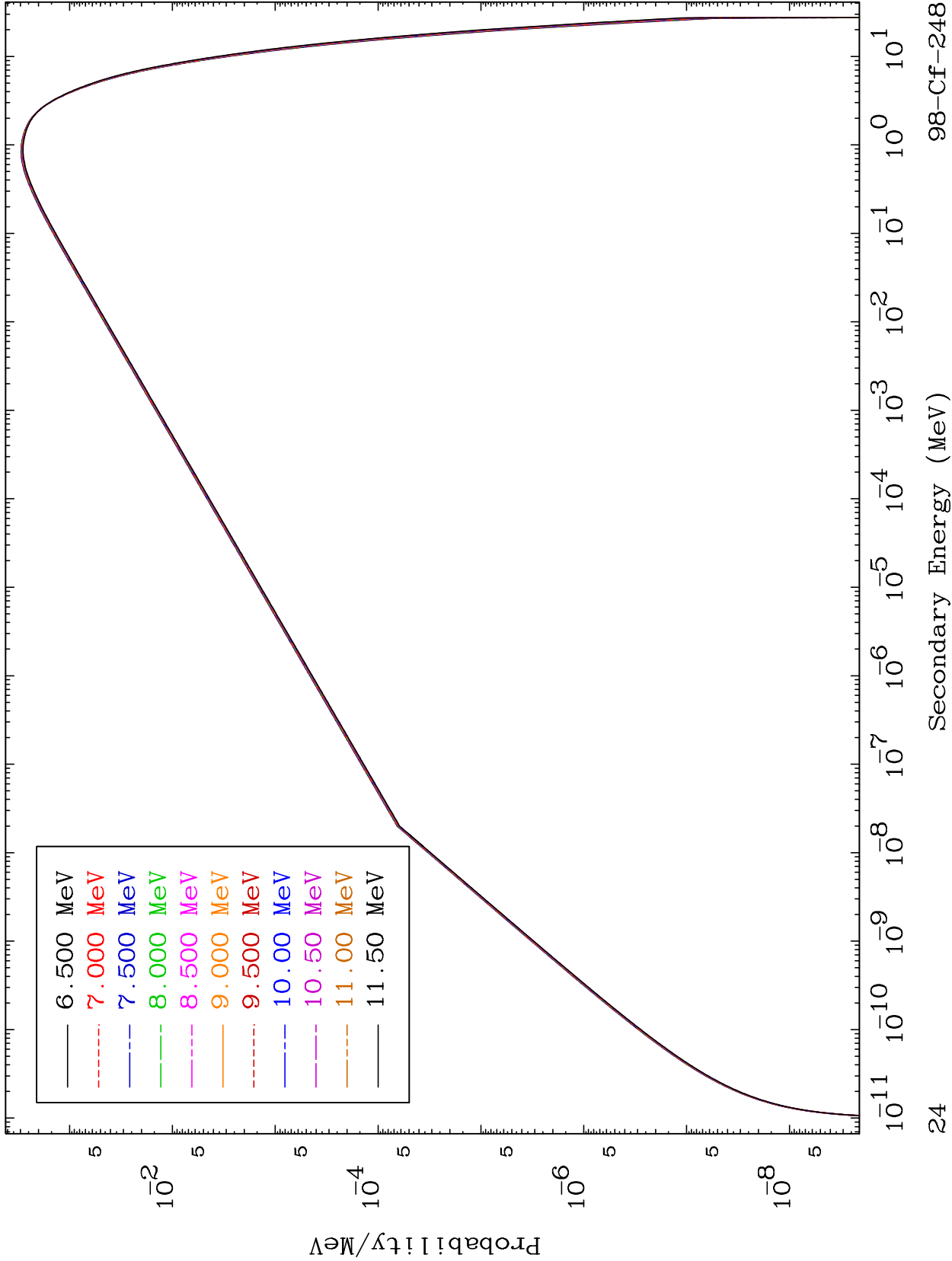
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Secondary Energy (MeV)

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Fission Energy Distributions

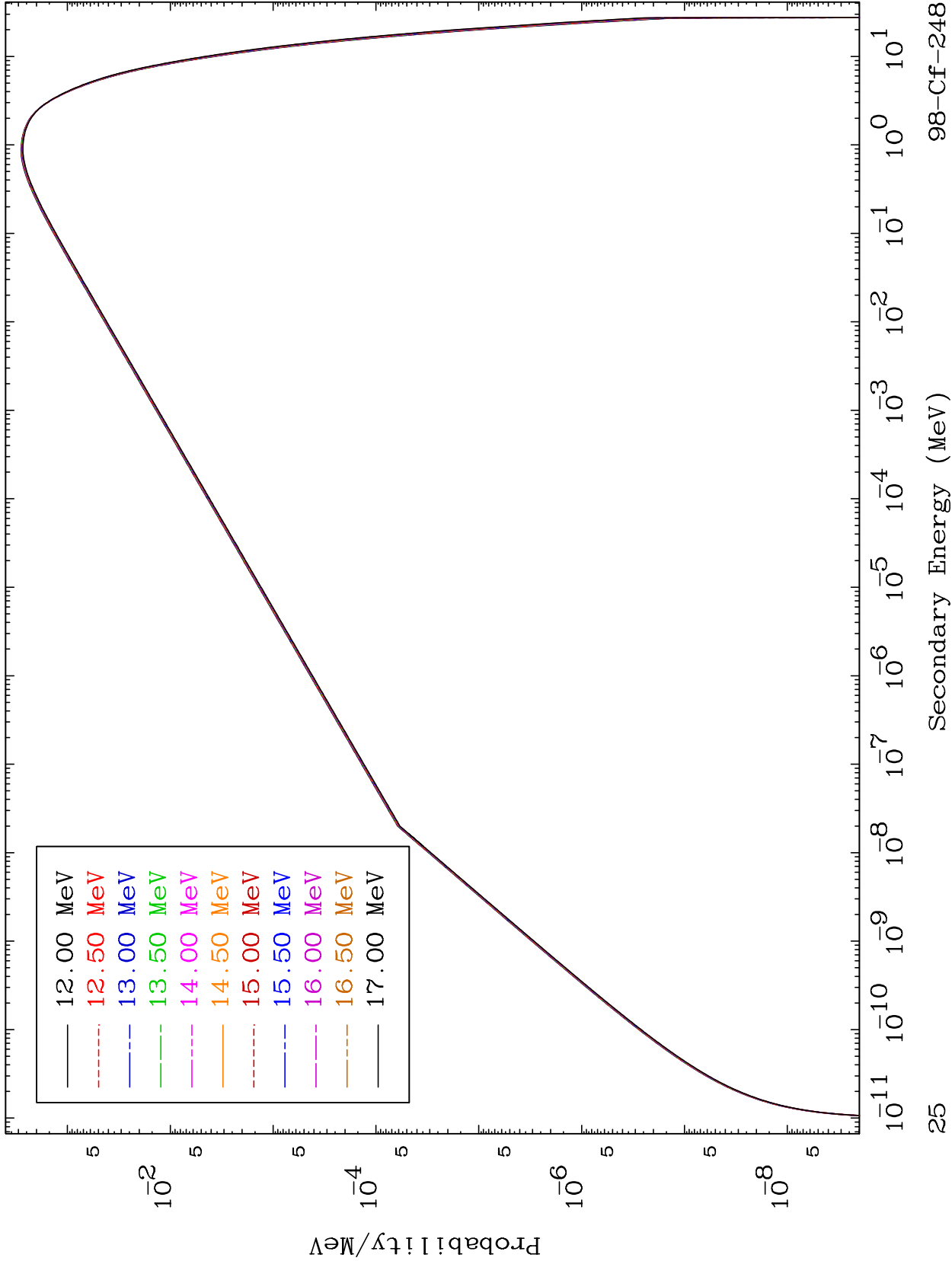
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Fission Energy Distributions

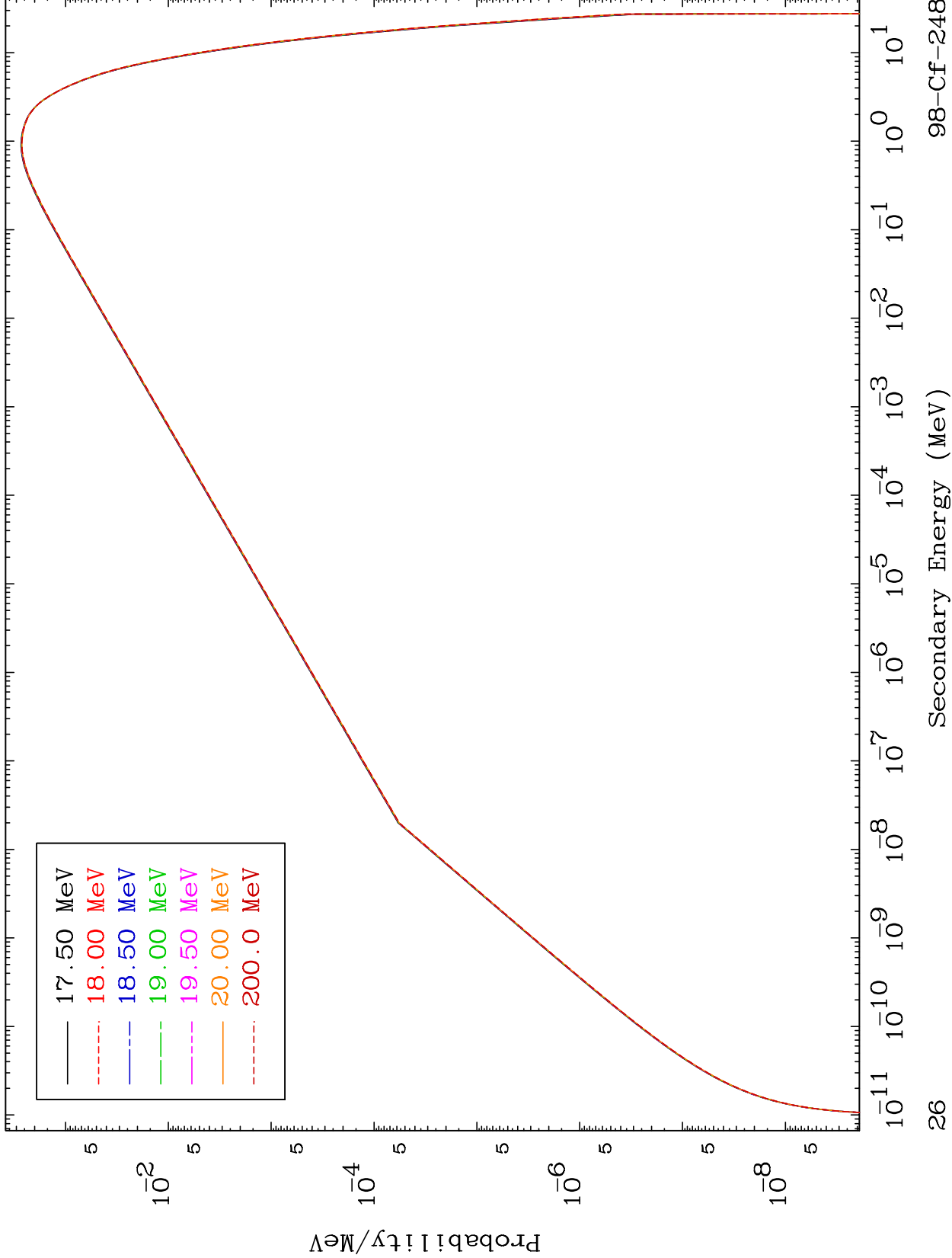
98-Cf-248



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Fission
Energy Distributions

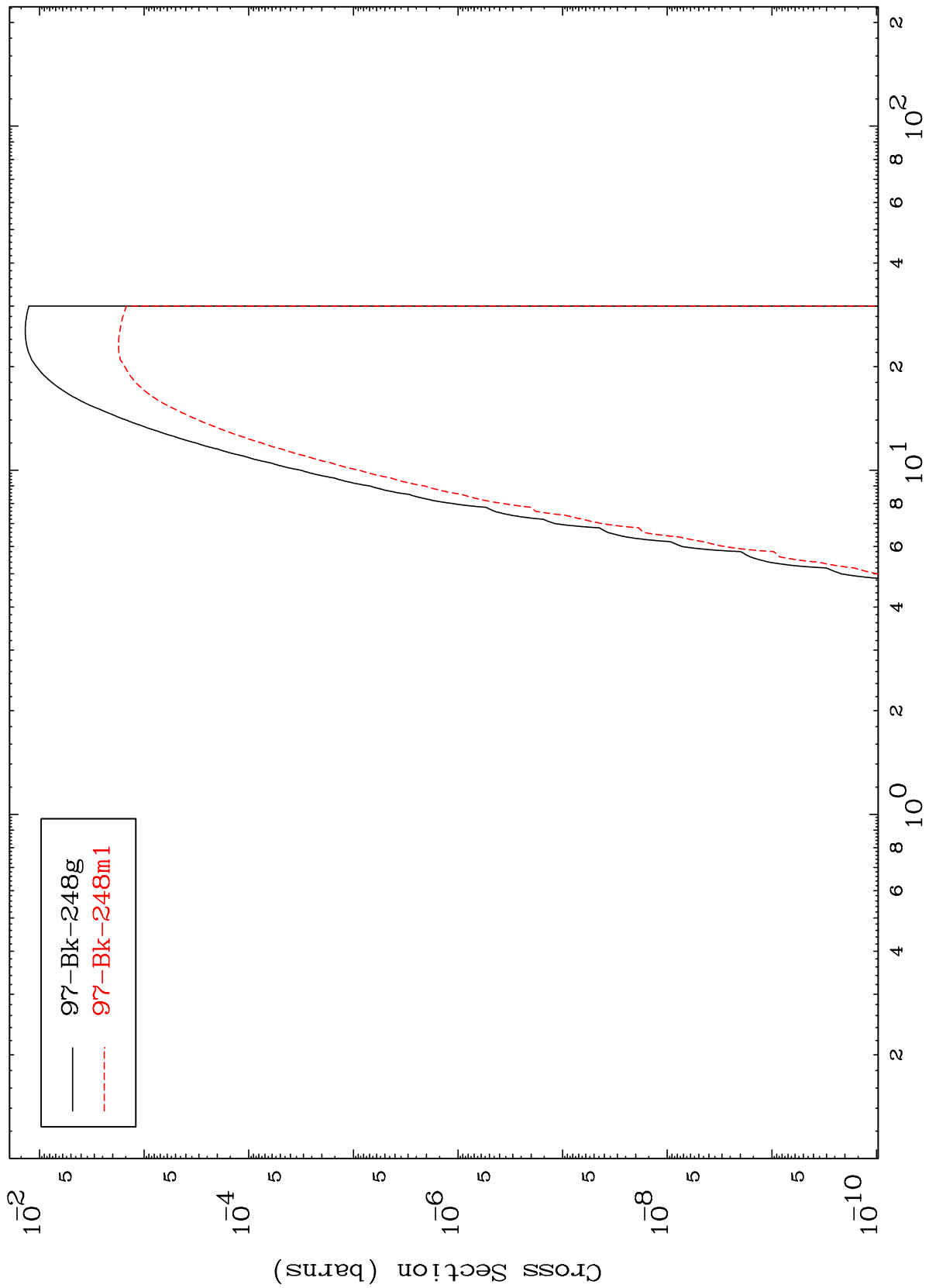
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98-Cf-248

(n,p)
Radionuclide Production Cross Section



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Incident Energy (MeV)

98-Cf-248

MAT 9849

(n,p) α

98-Cf-248

Radionuclide Production Cross Section

