

Program EVALPLOT  
(Version 2021-1)

by

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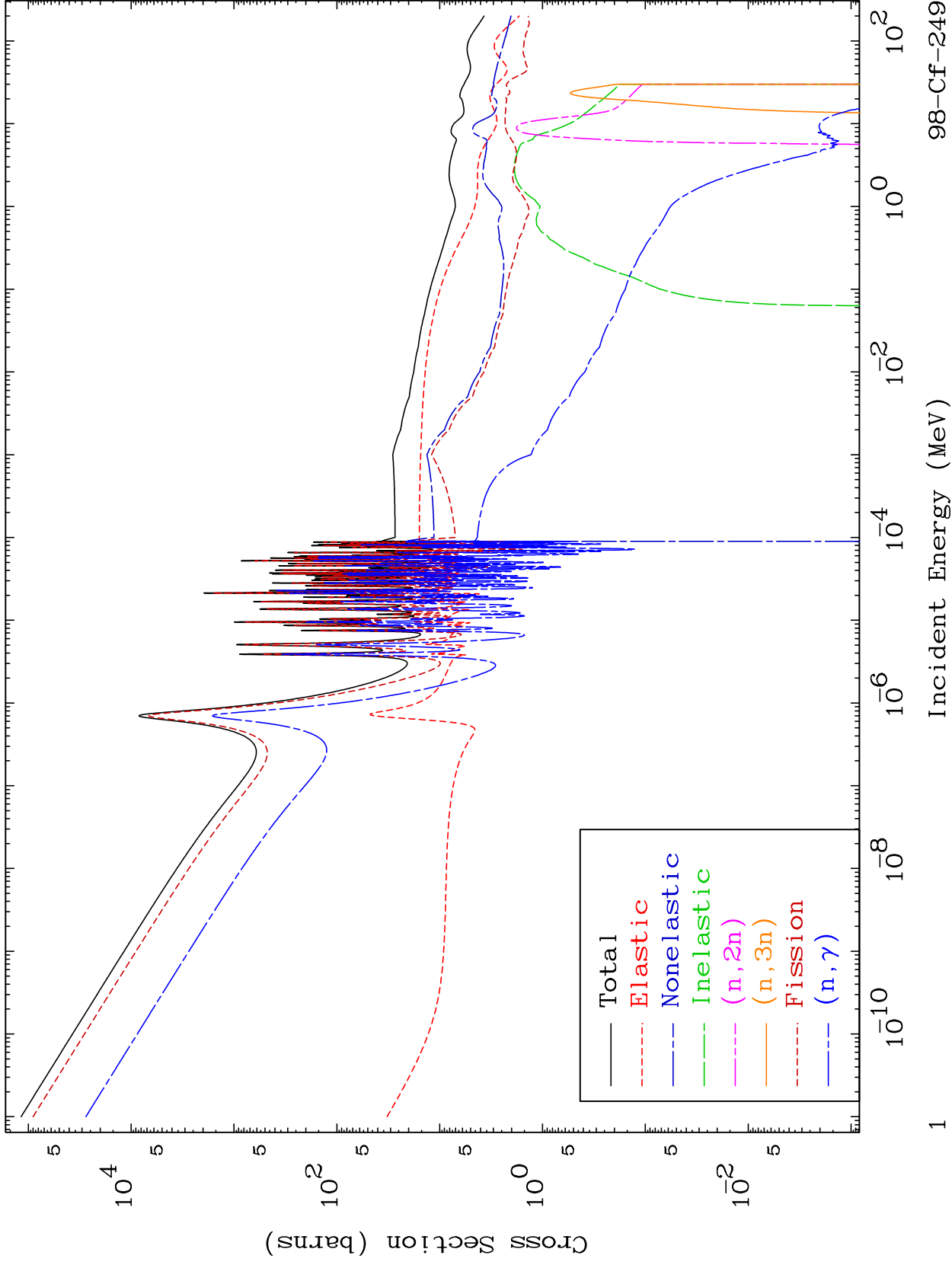
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9852

Neutron Major  
293 Kelvin Cross Sections

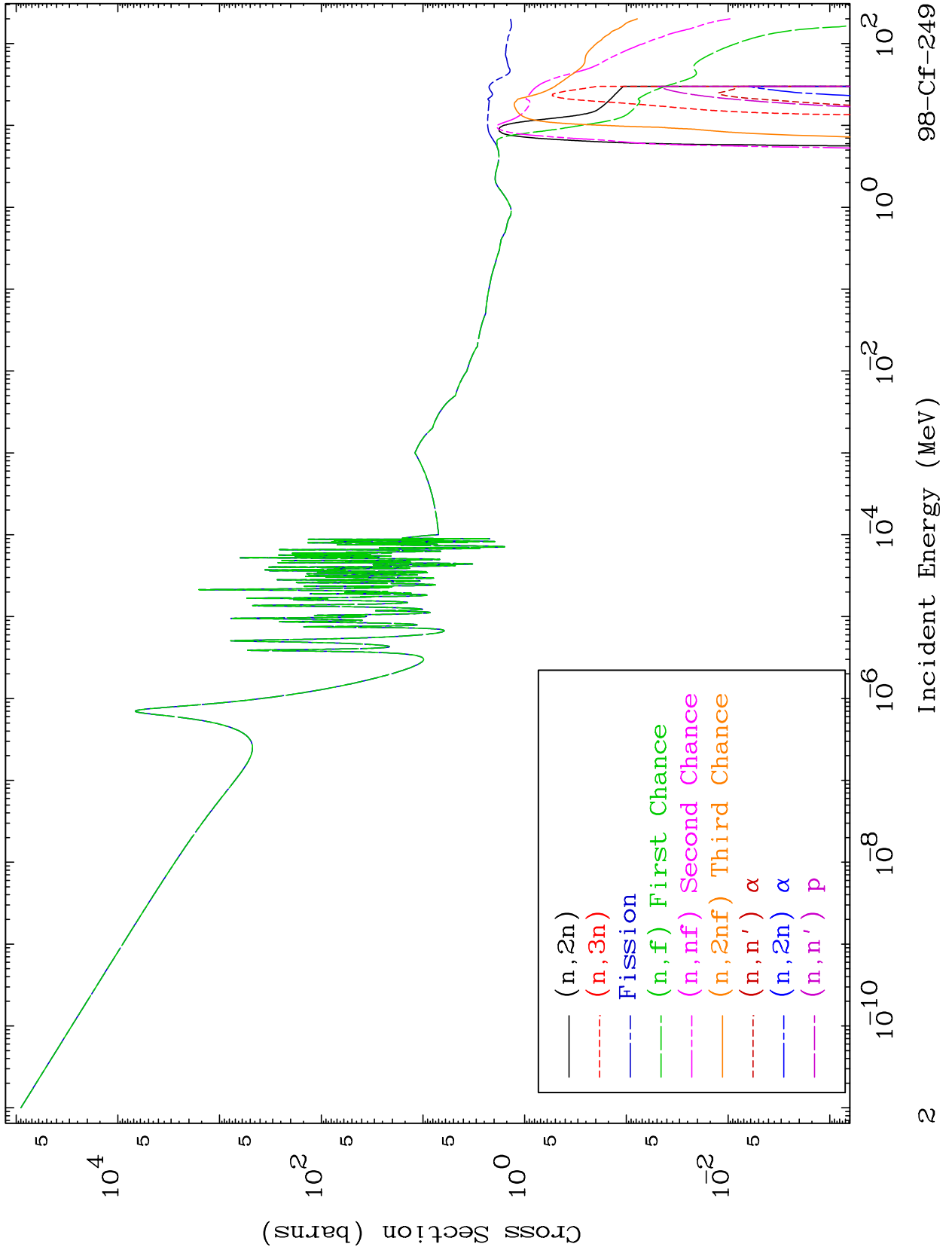
98-Cf-249

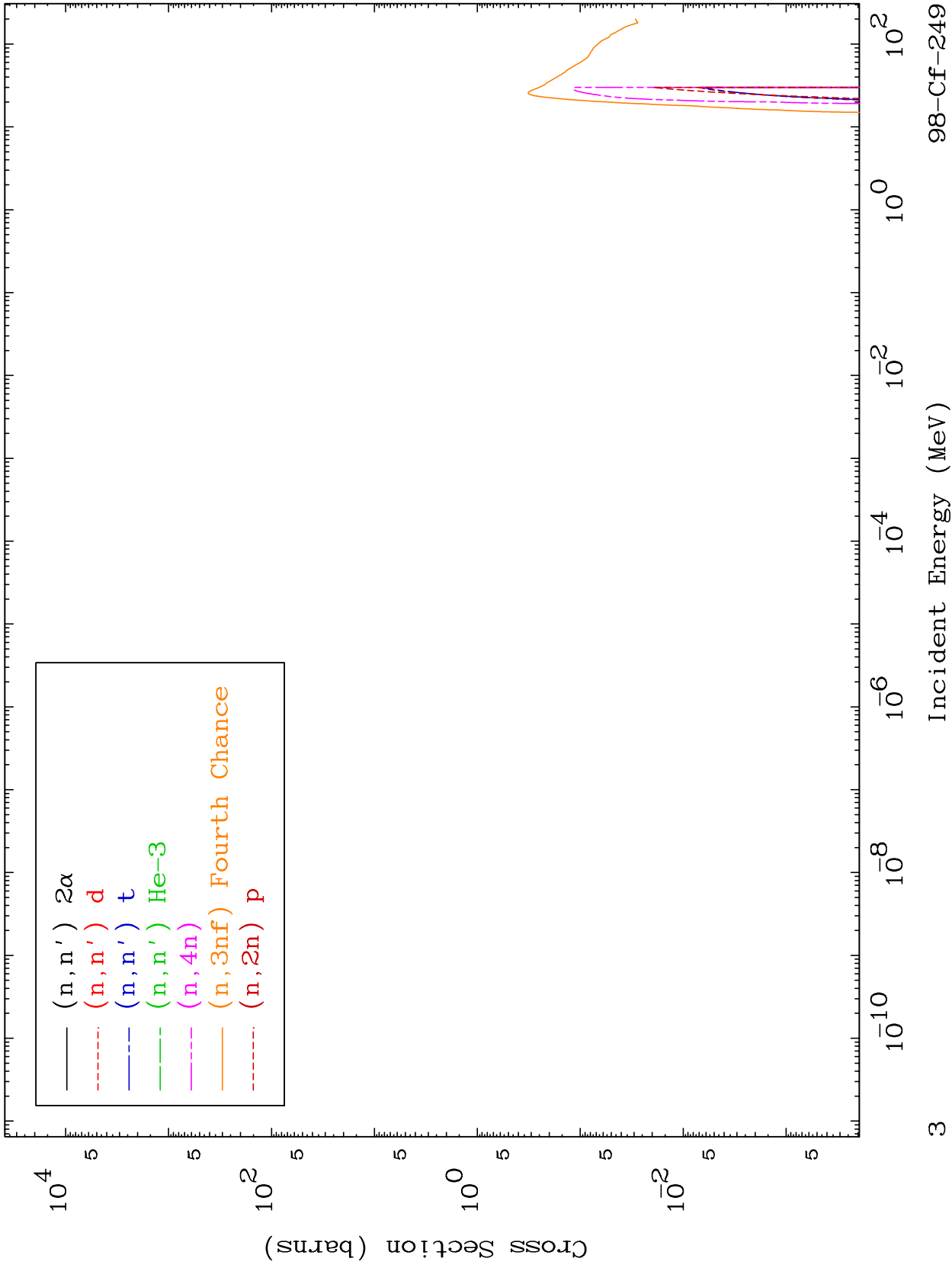


MAT 9852

Neutron Absorption  
293 Kelvin Cross Sections

98-Cf-249

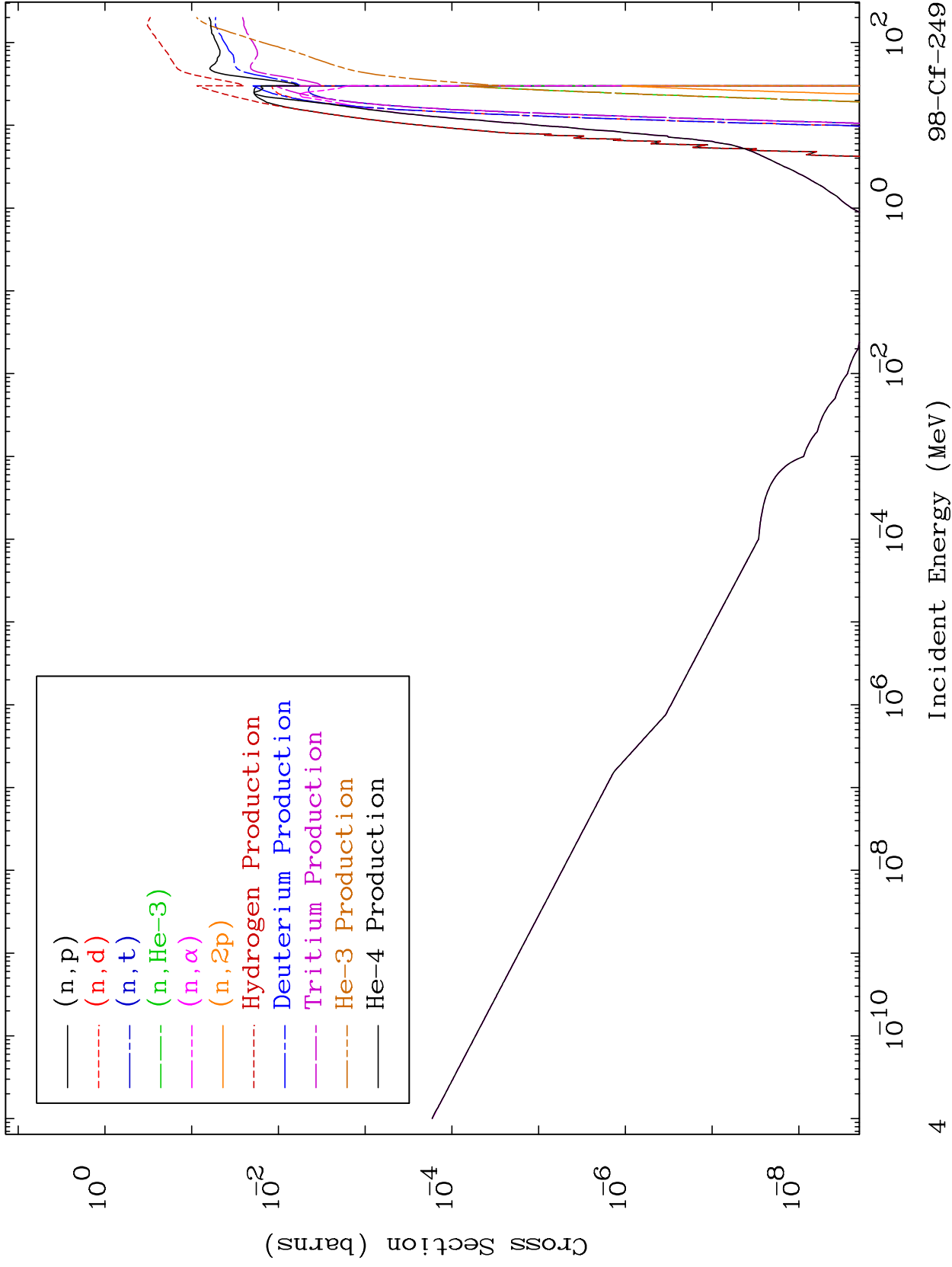




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Neutron Absorption  
293 Kelvin Cross Sections

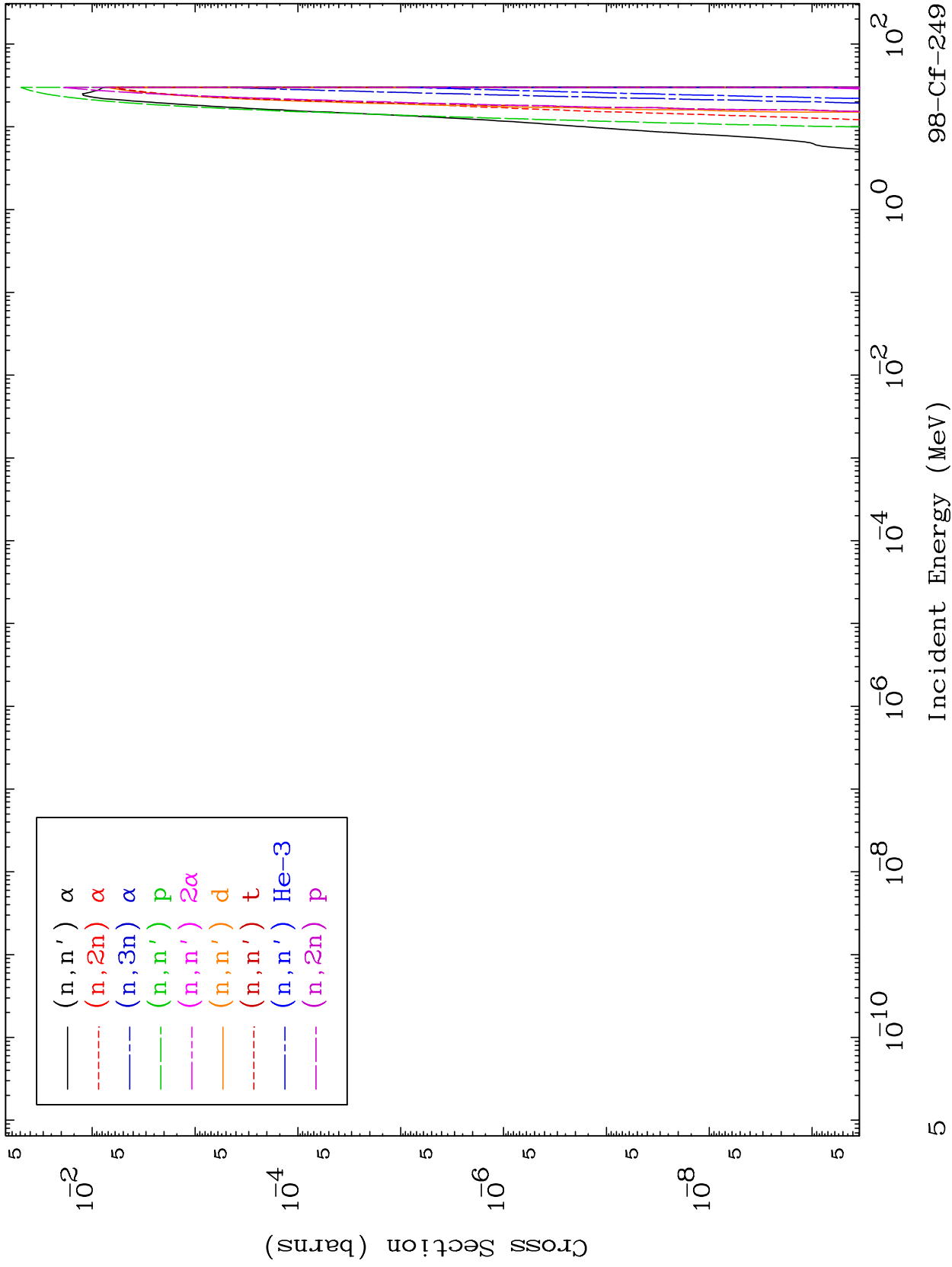
98-Cf-249



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Charged Particle  
293 Kelvin Cross Sections

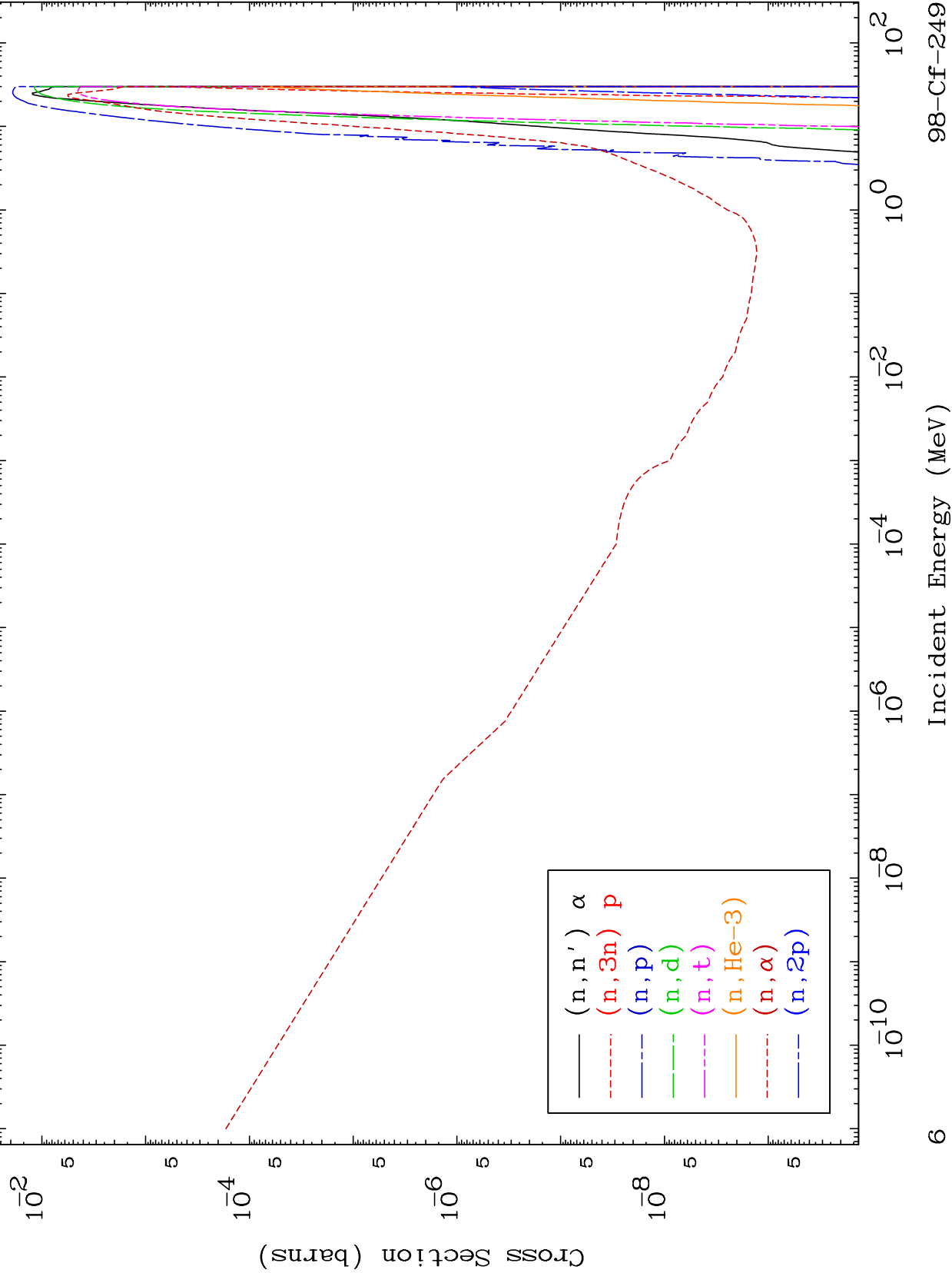
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Charged Particle  
293 Kelvin Cross Sections

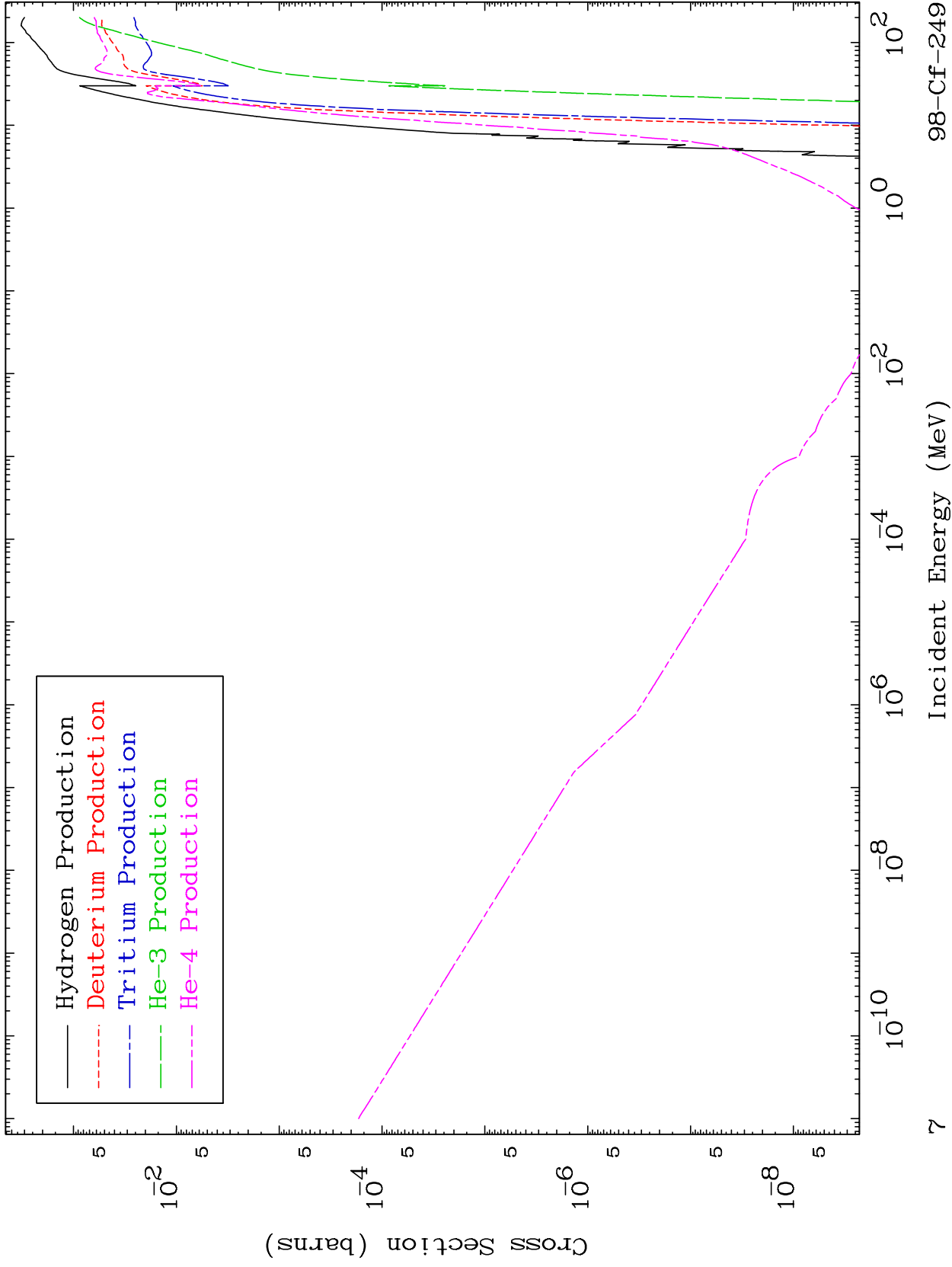
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Particle Production  
293 Kelvin Cross Sections

98-Cf-249

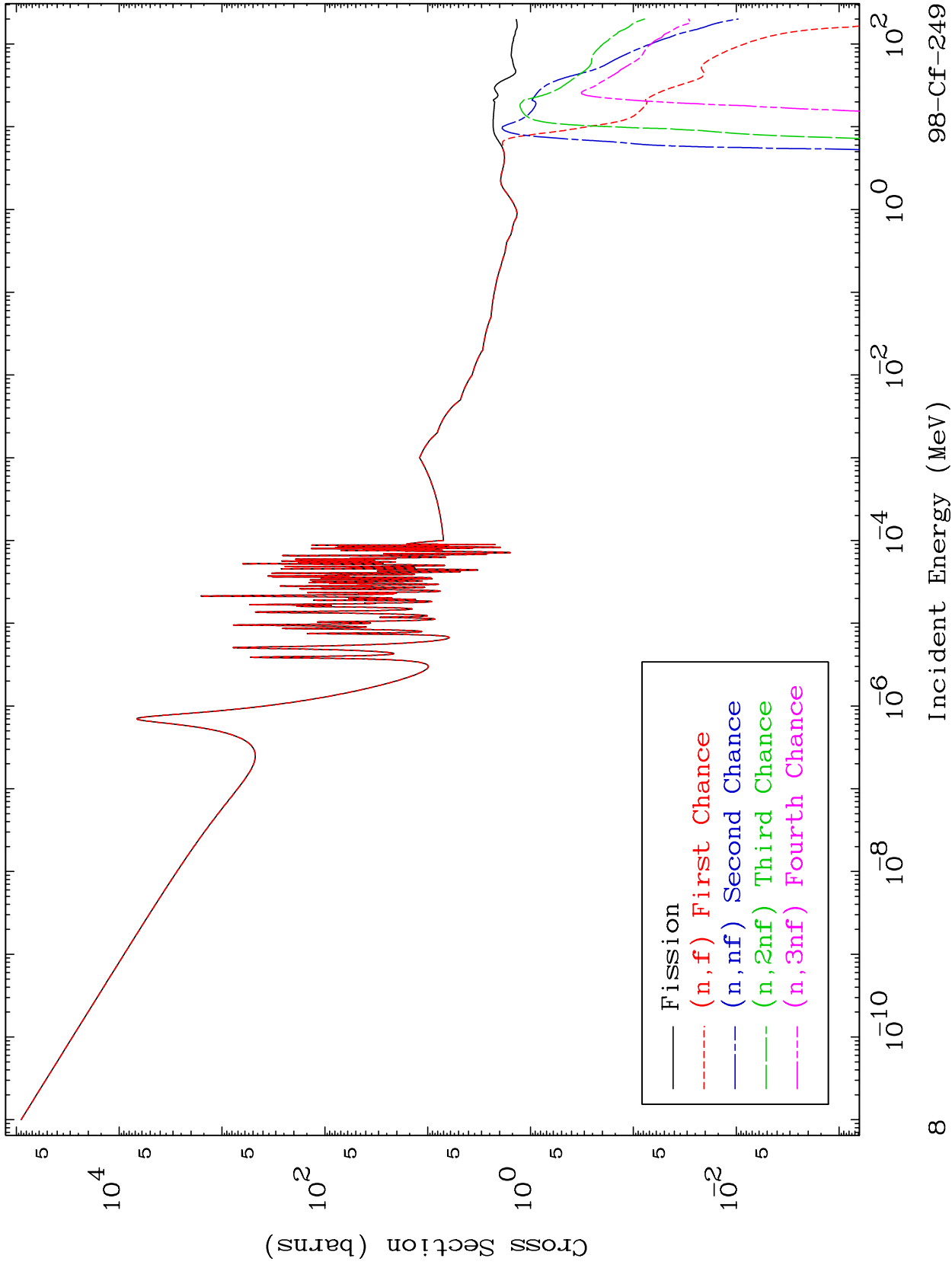




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Fission  
293 Kelvin Cross Sections

98-Cf-249

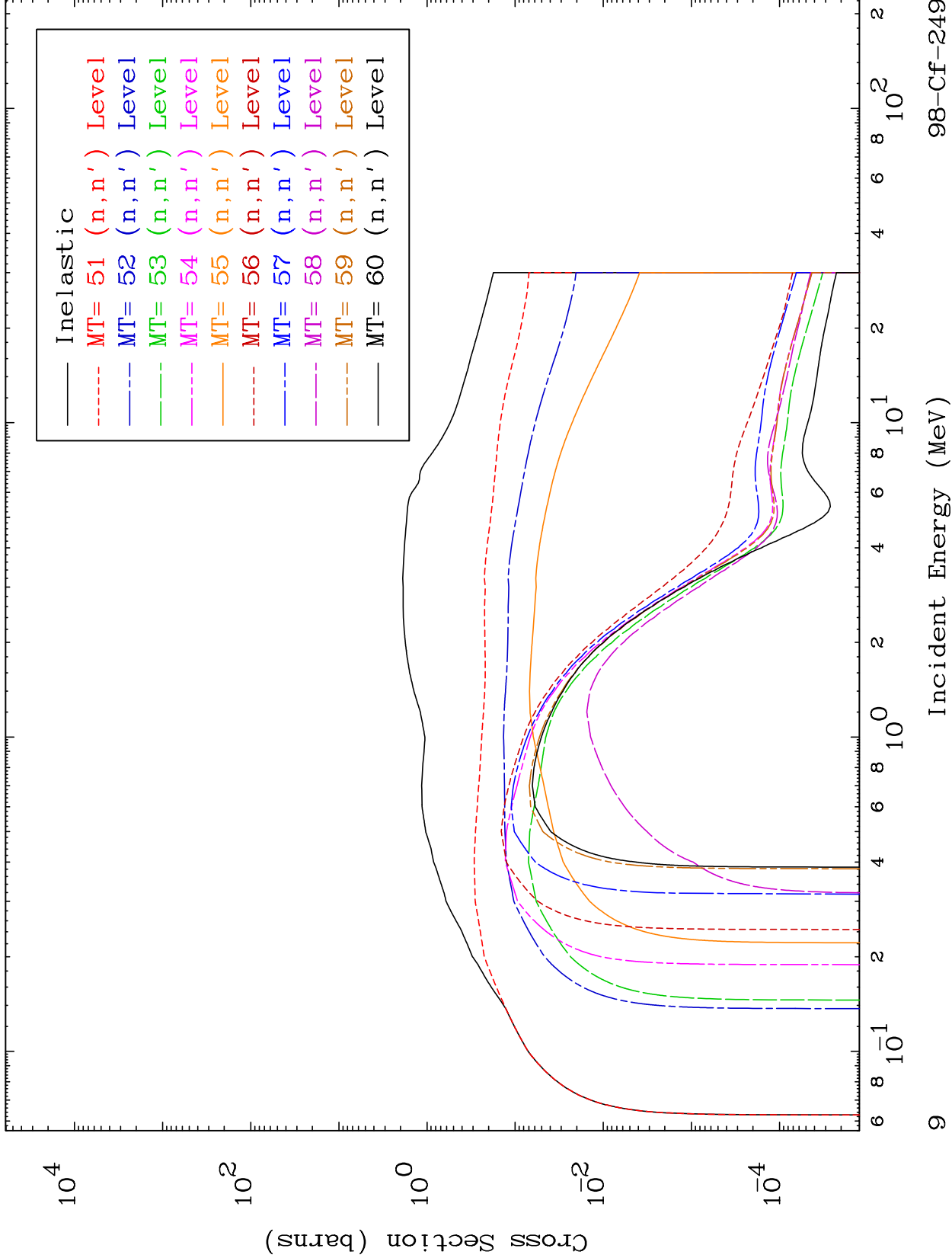


MAT 9852

(n,n') Levels

98-Cf-249

293 Kelvin Cross Sections



9

Incident Energy (MeV)

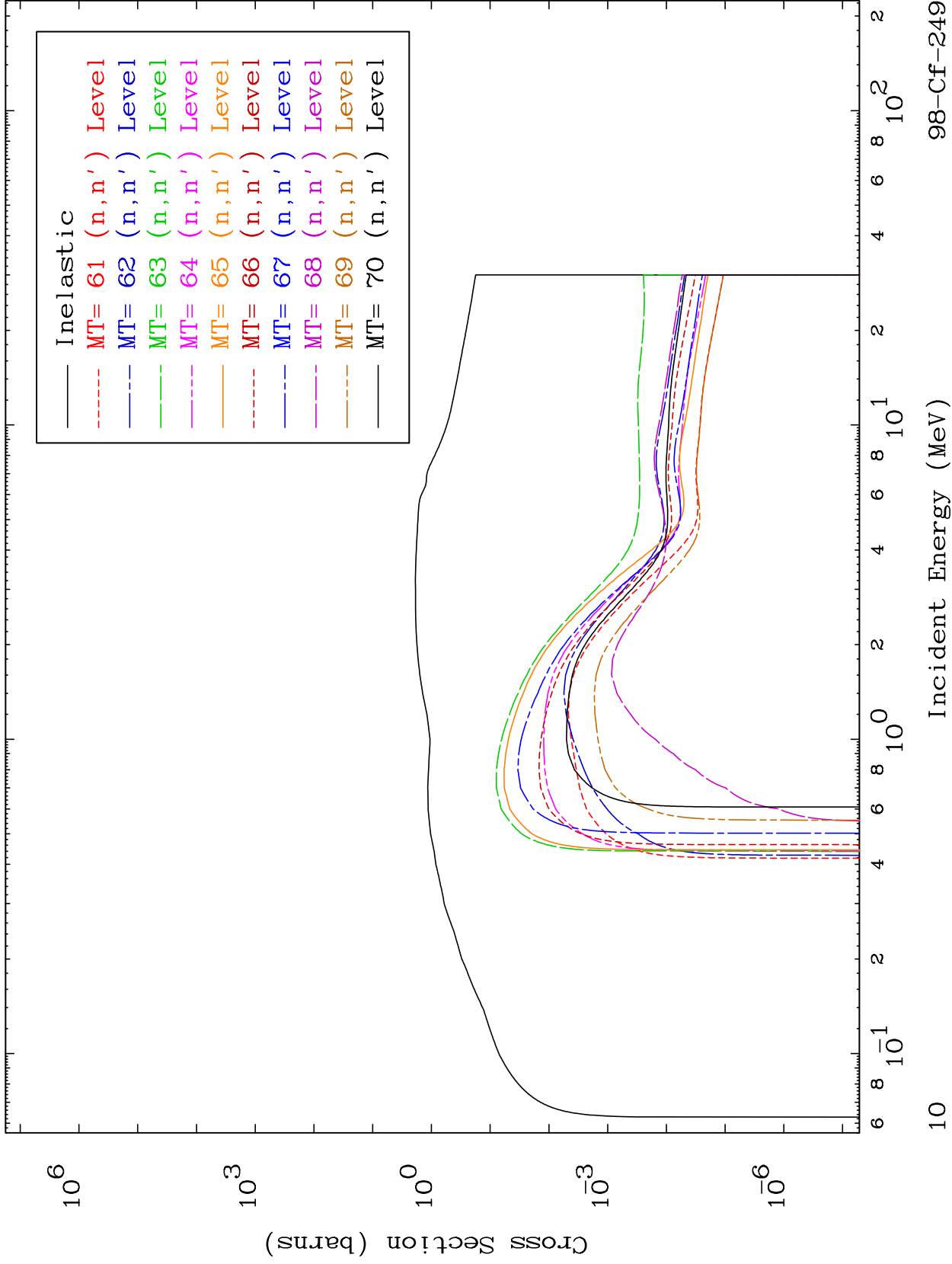
98-Cf-249

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(n,n') Levels

98-Cf-249

293 Kelvin Cross Sections



10

Incident Energy (MeV)

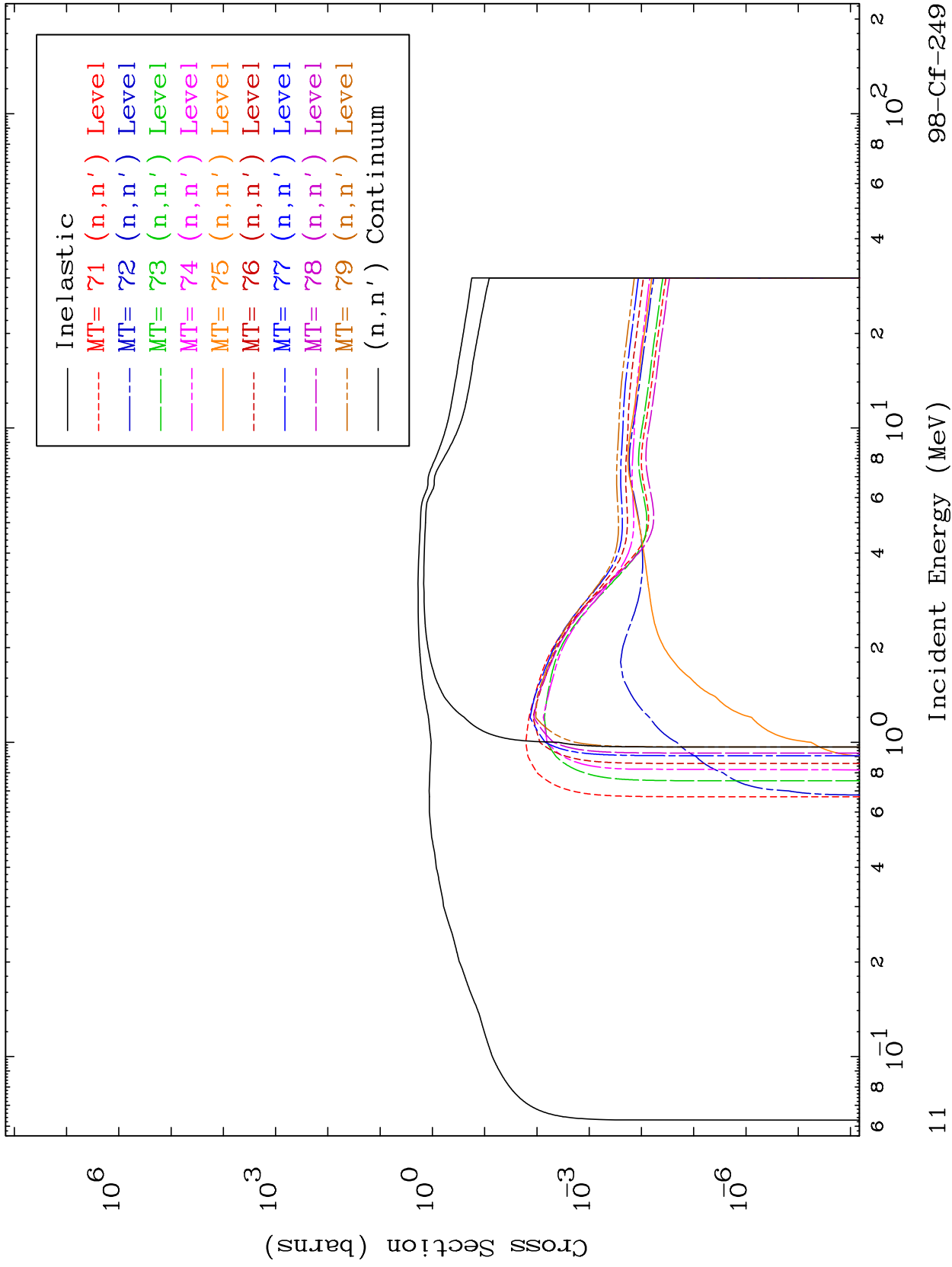
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(n,n') Levels

98-Cf-249

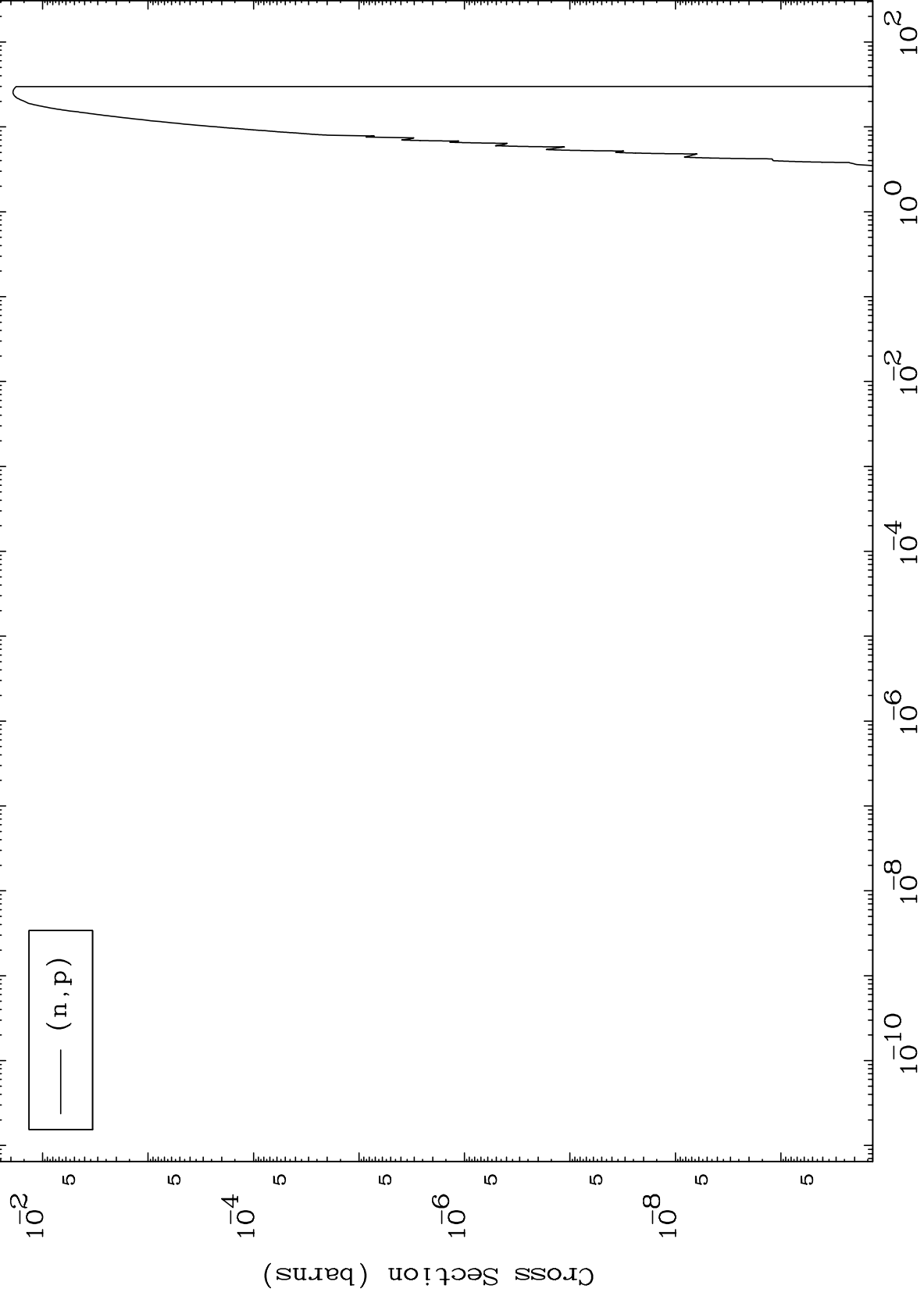
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

98-Cf-249



12

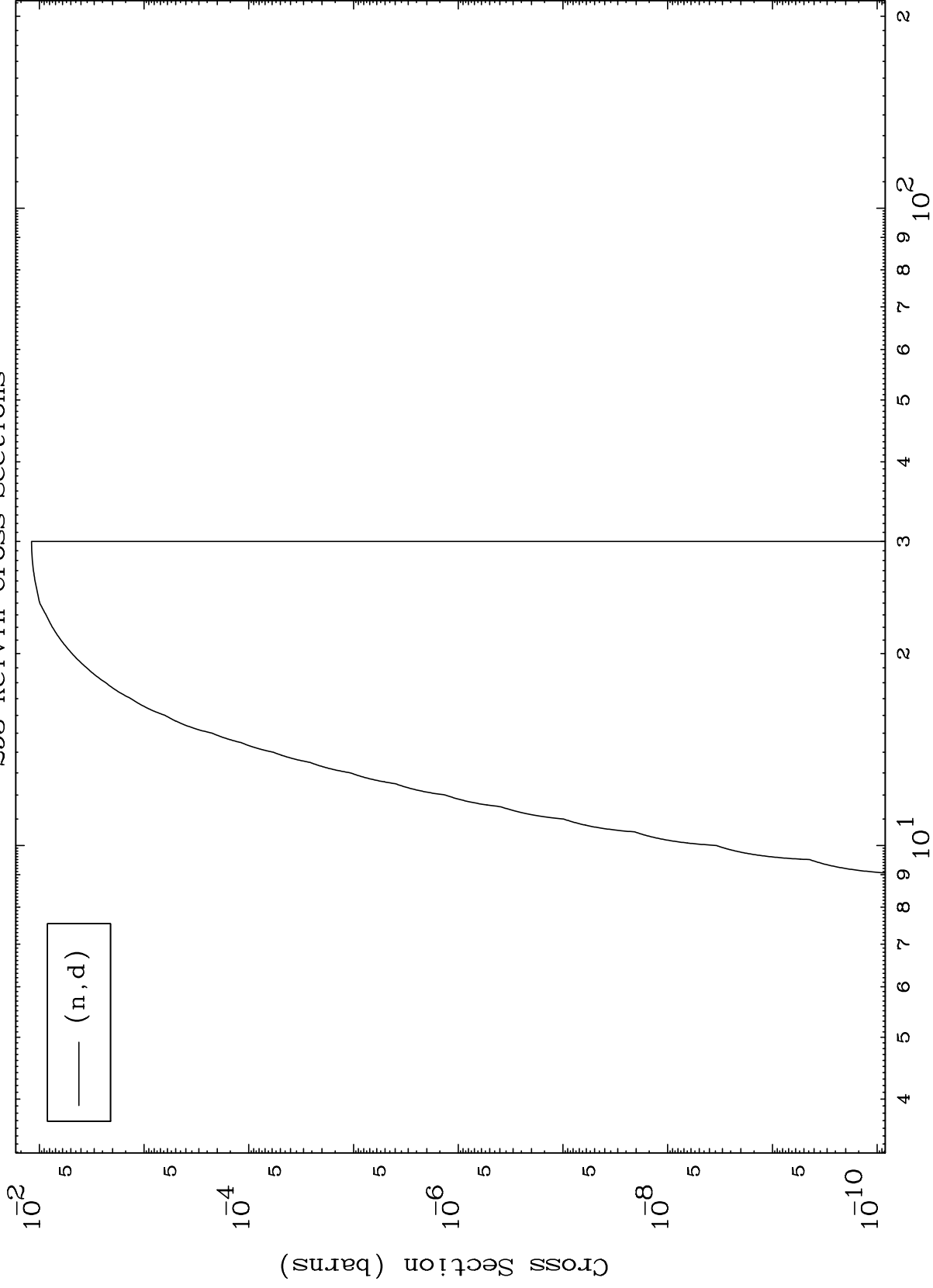
Incident Energy (MeV)

98-Cf-249

MAT 9852

(n,d) Levels  
293 Kelvin Cross Sections

98-Cf-249



13

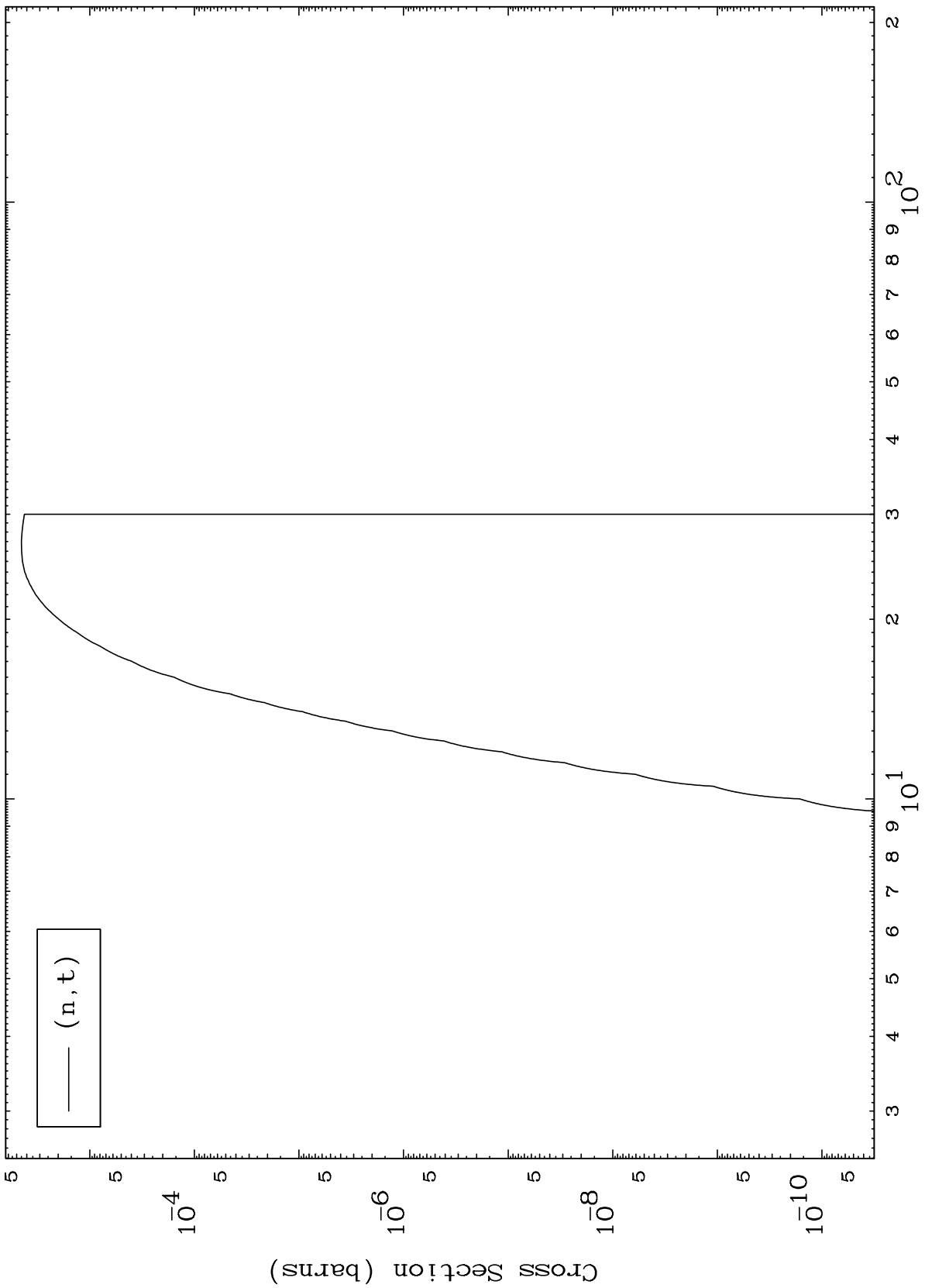
Incident Energy (MeV)

98-Cf-249

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(n,t) Levels  
293 Kelvin Cross Sections

98-Cf-249



14

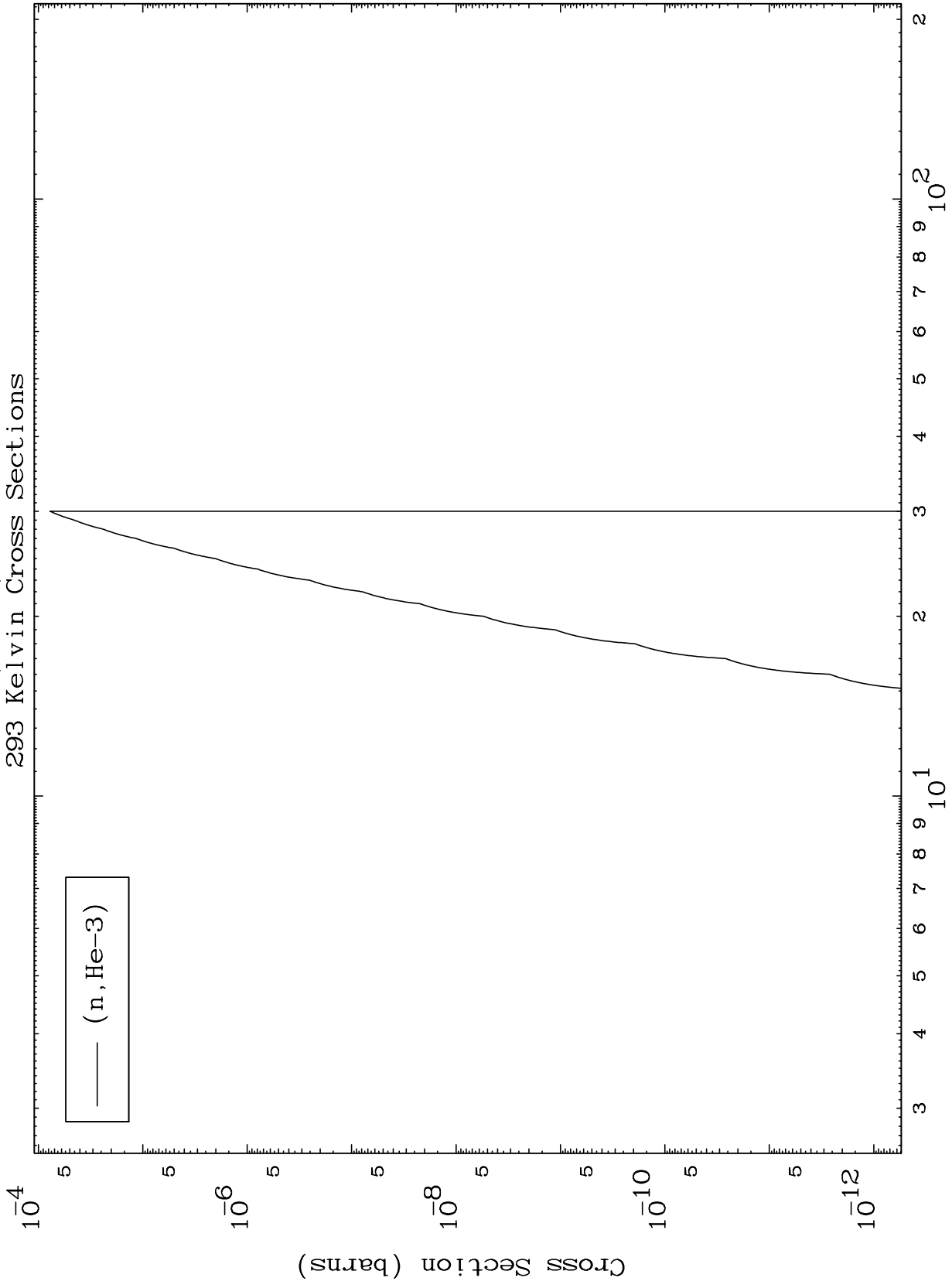
Incident Energy (MeV)

98-Cf-249

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(n,He3) Levels  
293 Kelvin Cross Sections

98-Cf-249



15

Incident Energy (MeV)

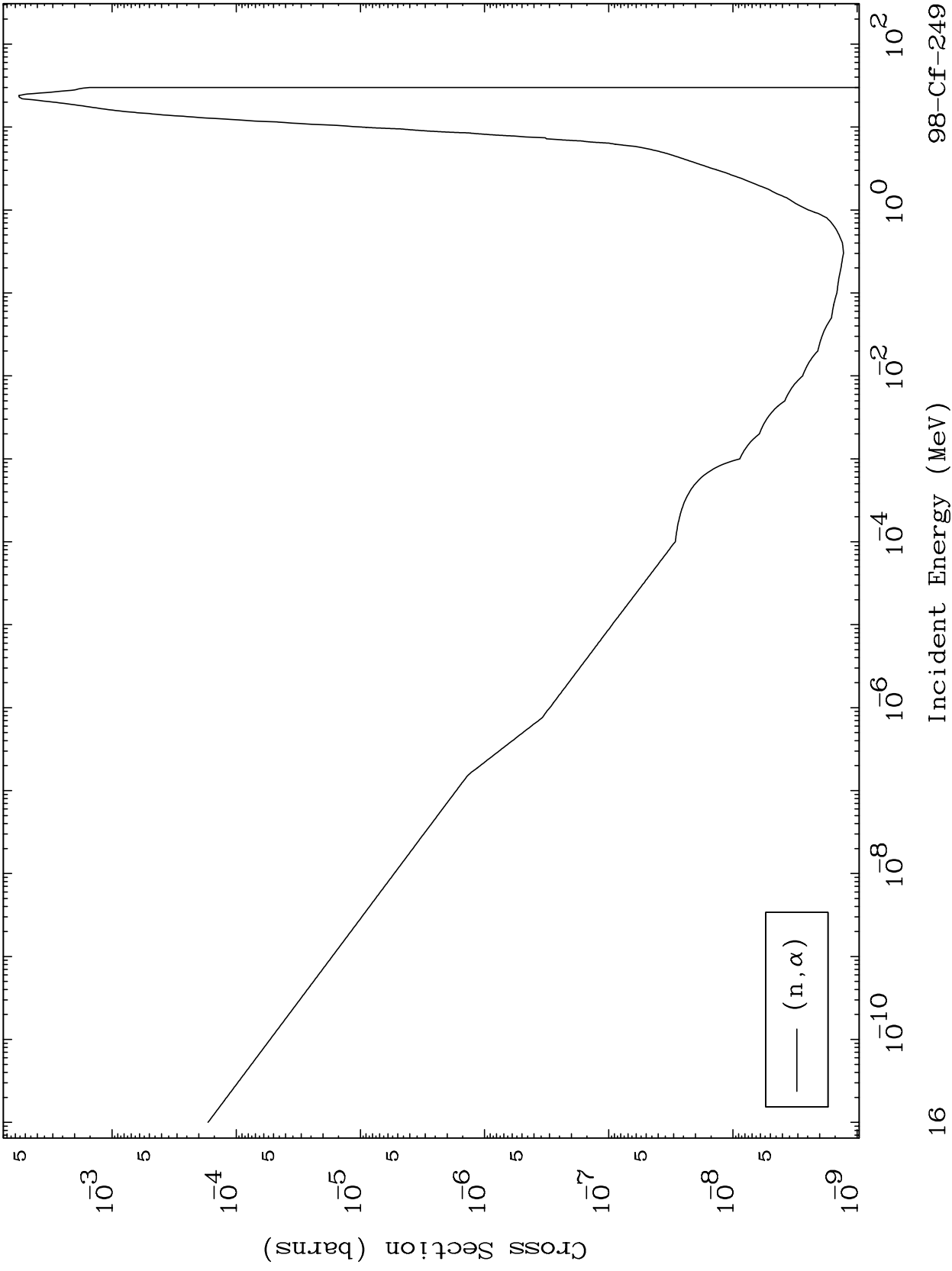
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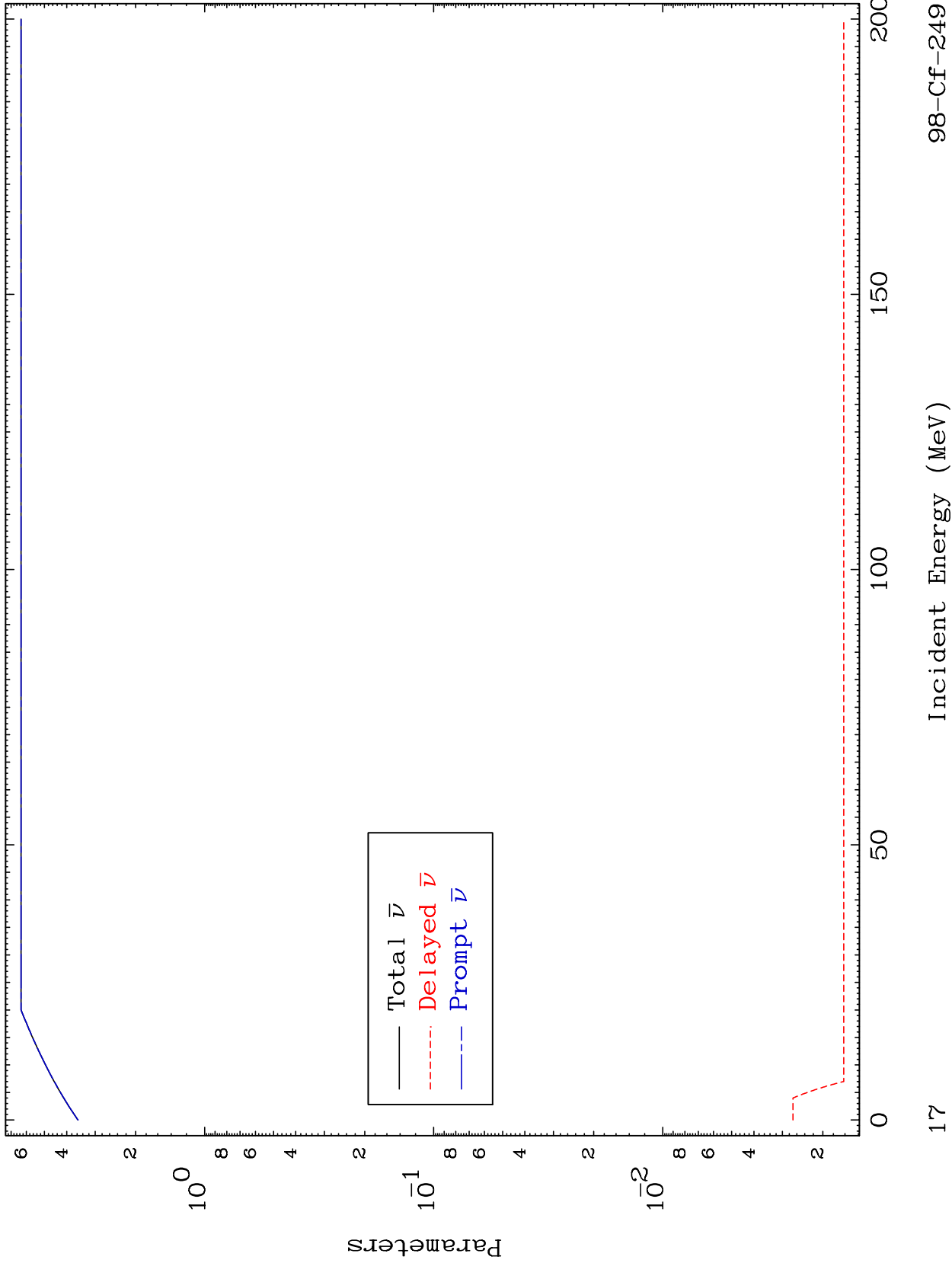


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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

98-Cf-249

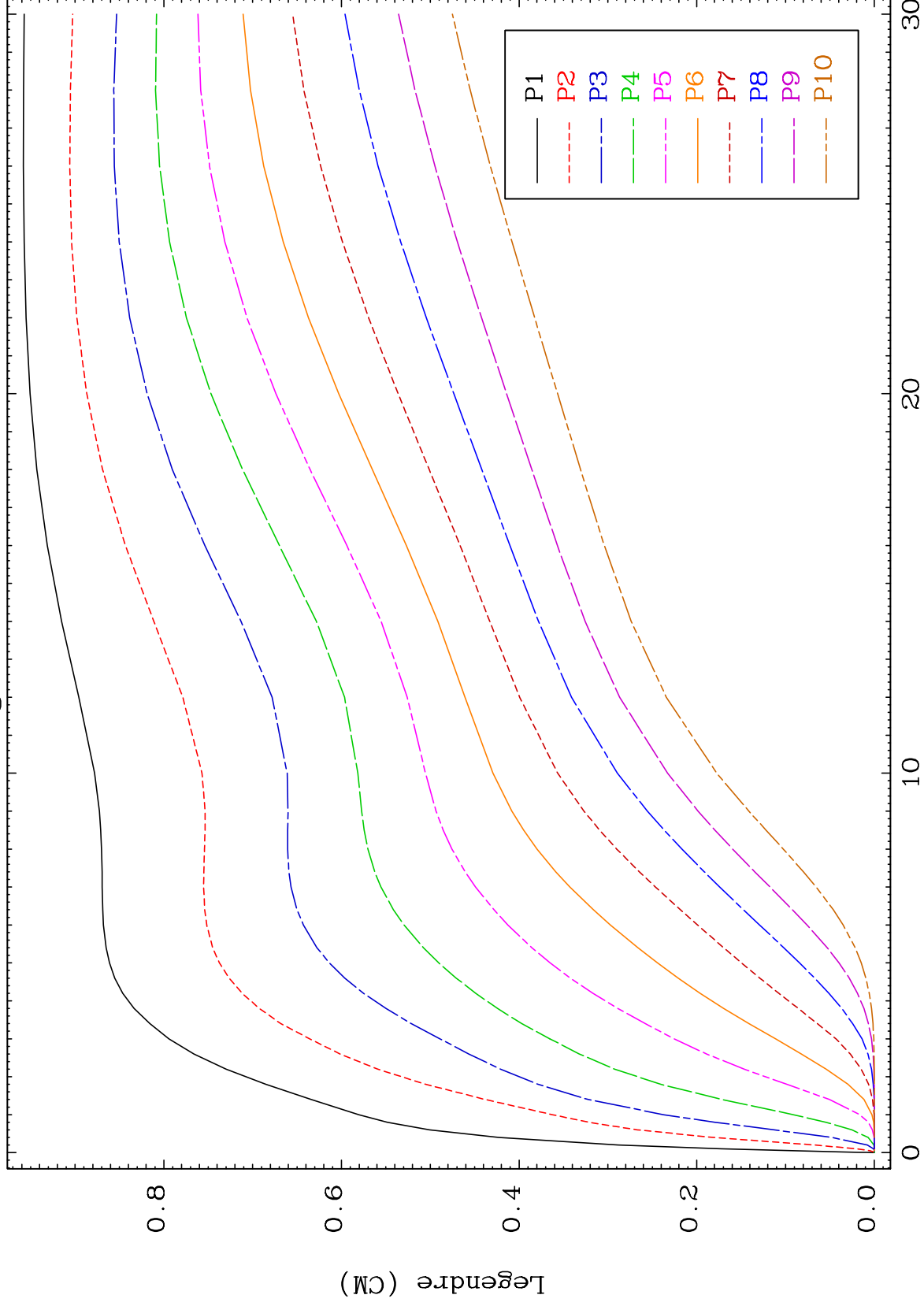




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Elastic Legendre Coefficients

98-Cf-249



18

Incident Energy (MeV)

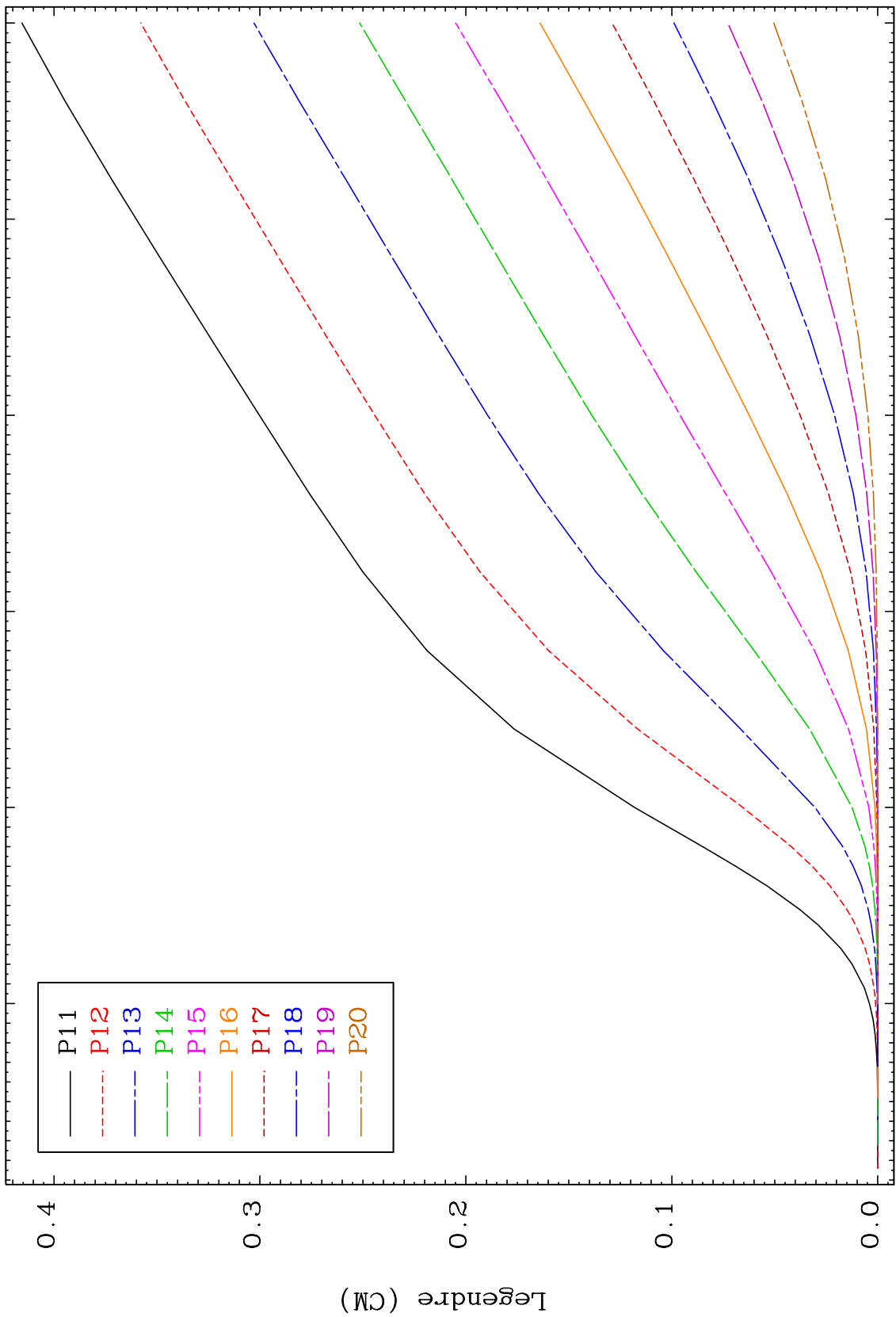
98-Cf-249

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Elastic Legendre Coefficients

98-Cf-249



19

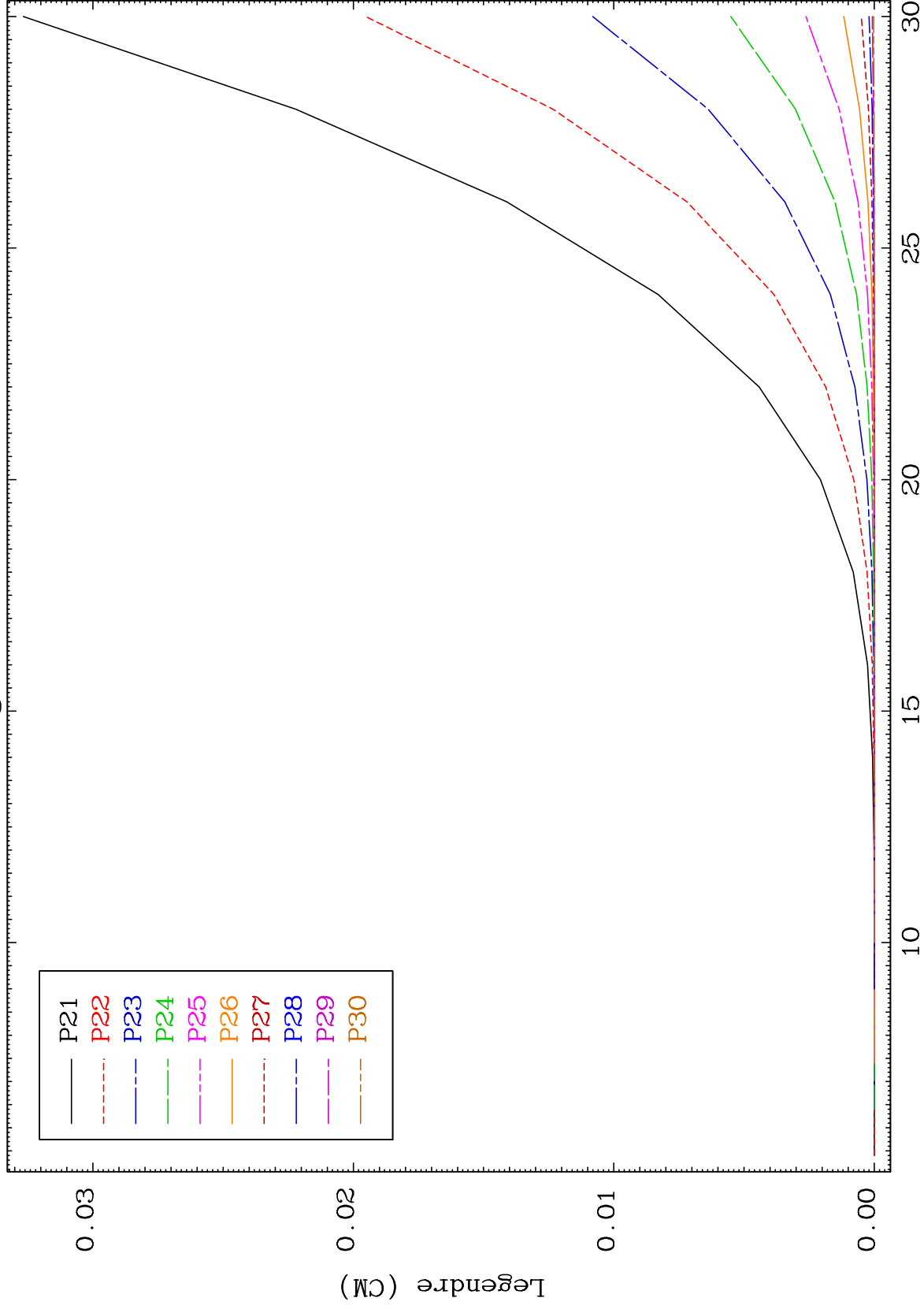
Incident Energy (MeV)

98-Cf-249

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### Elastic Legendre Coefficients

98-Cf-249



20

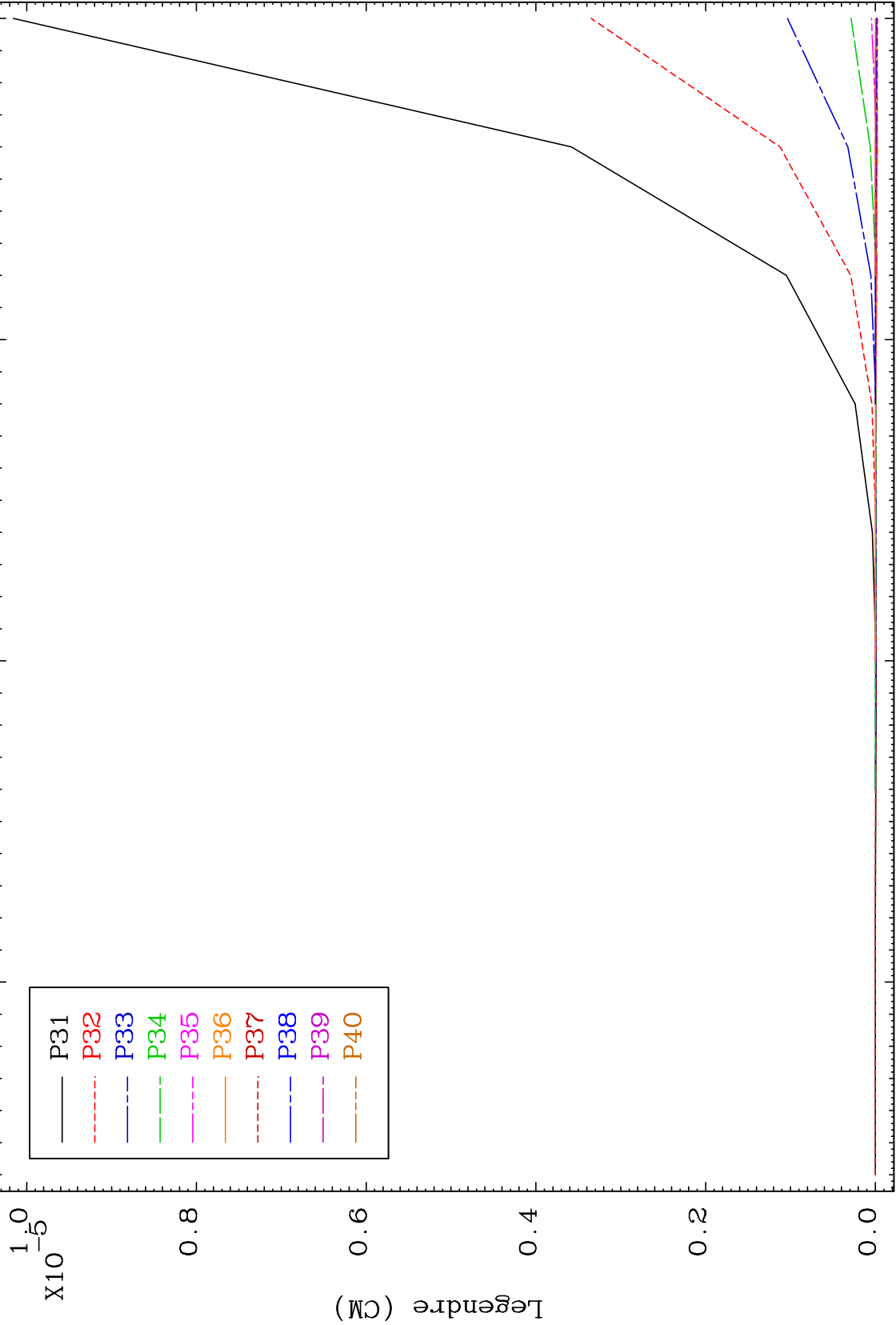
Incident Energy (MeV)

98-Cf-249

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Elastic Legendre Coefficients

98-Cf-249



21

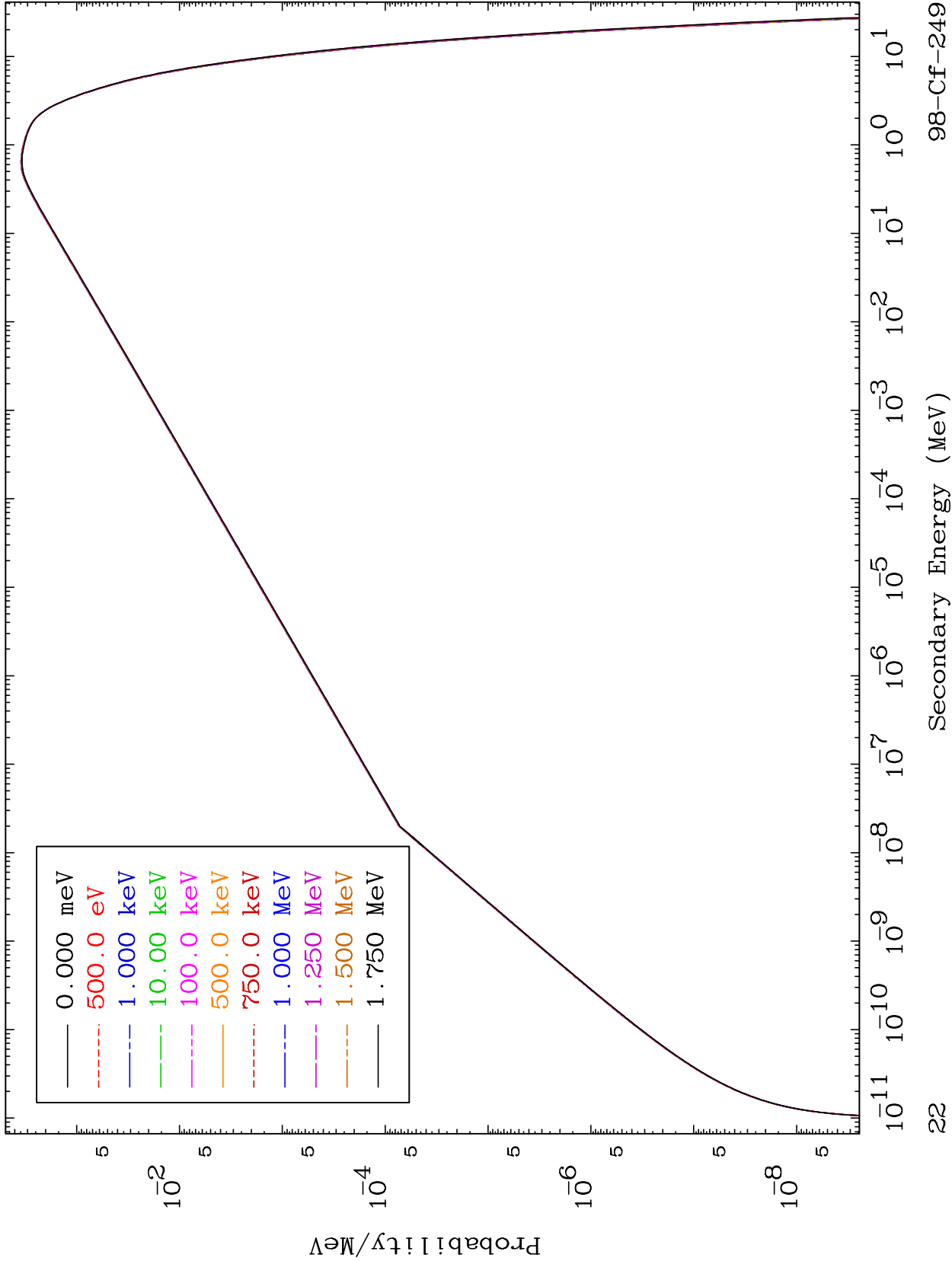
Incident Energy (MeV)

98-Cf-249

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### Fission Energy Distributions

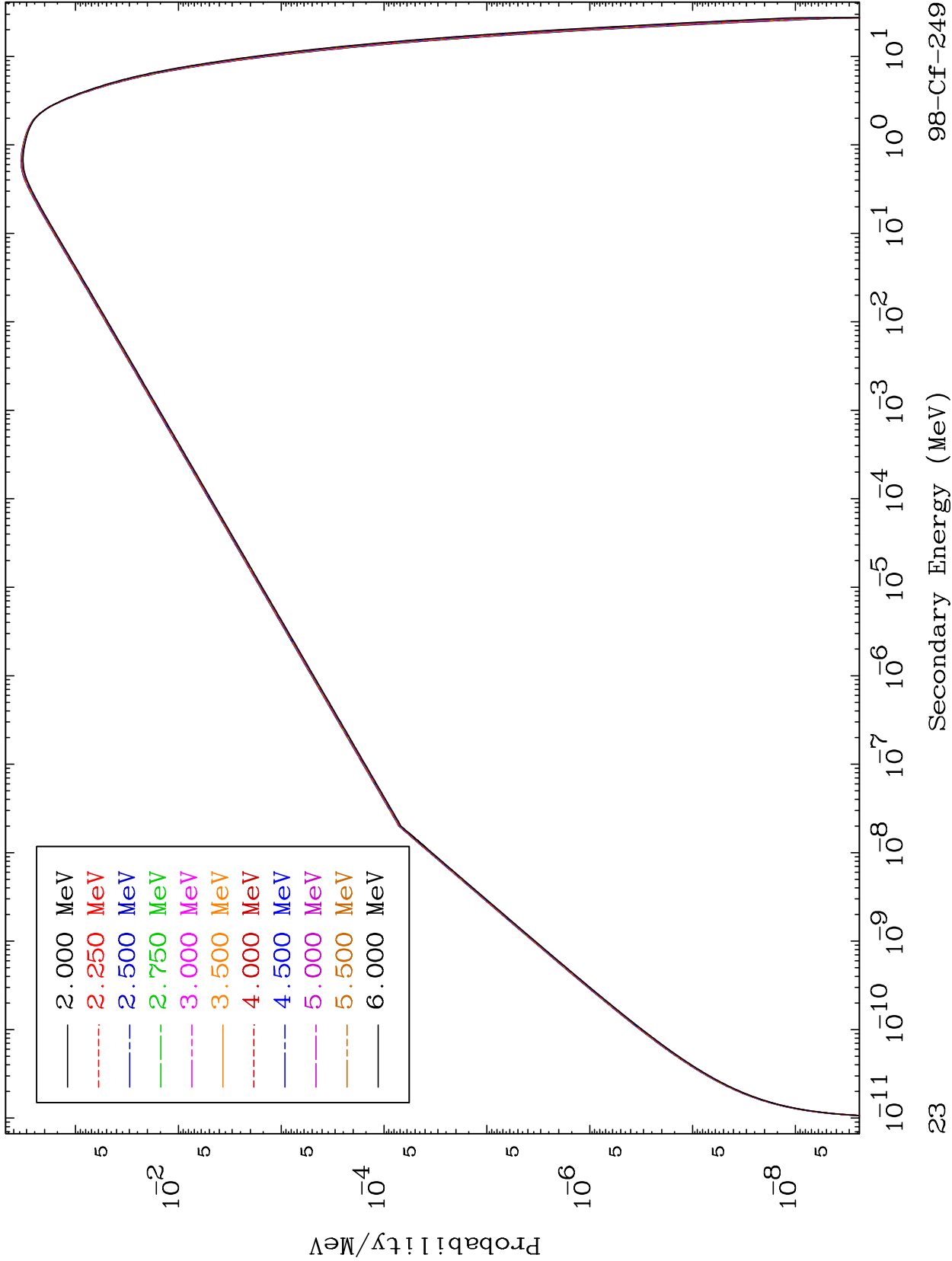
98-Cf-249



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### Fission Energy Distributions

98-Cf-249

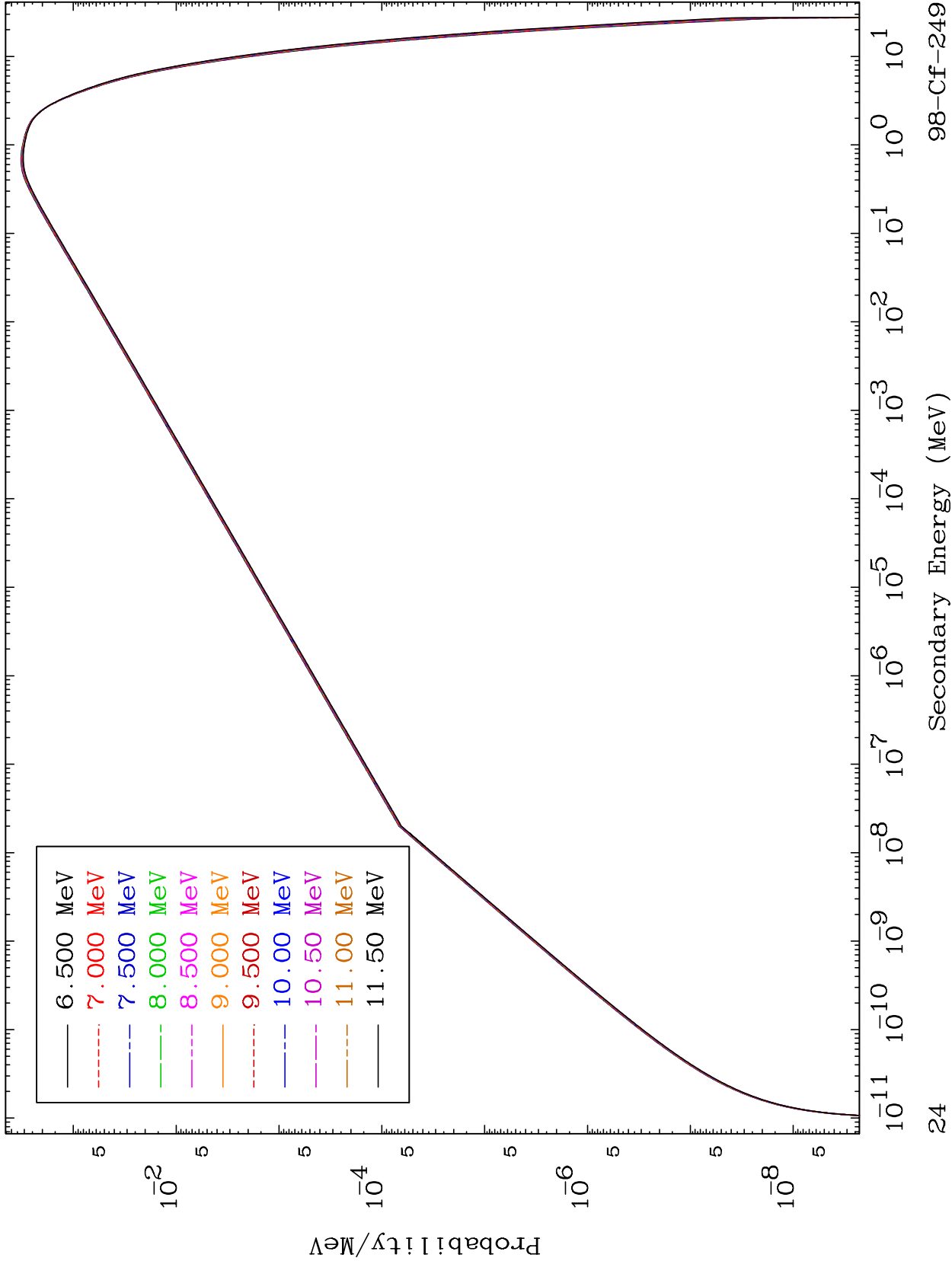




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### Fission Energy Distributions

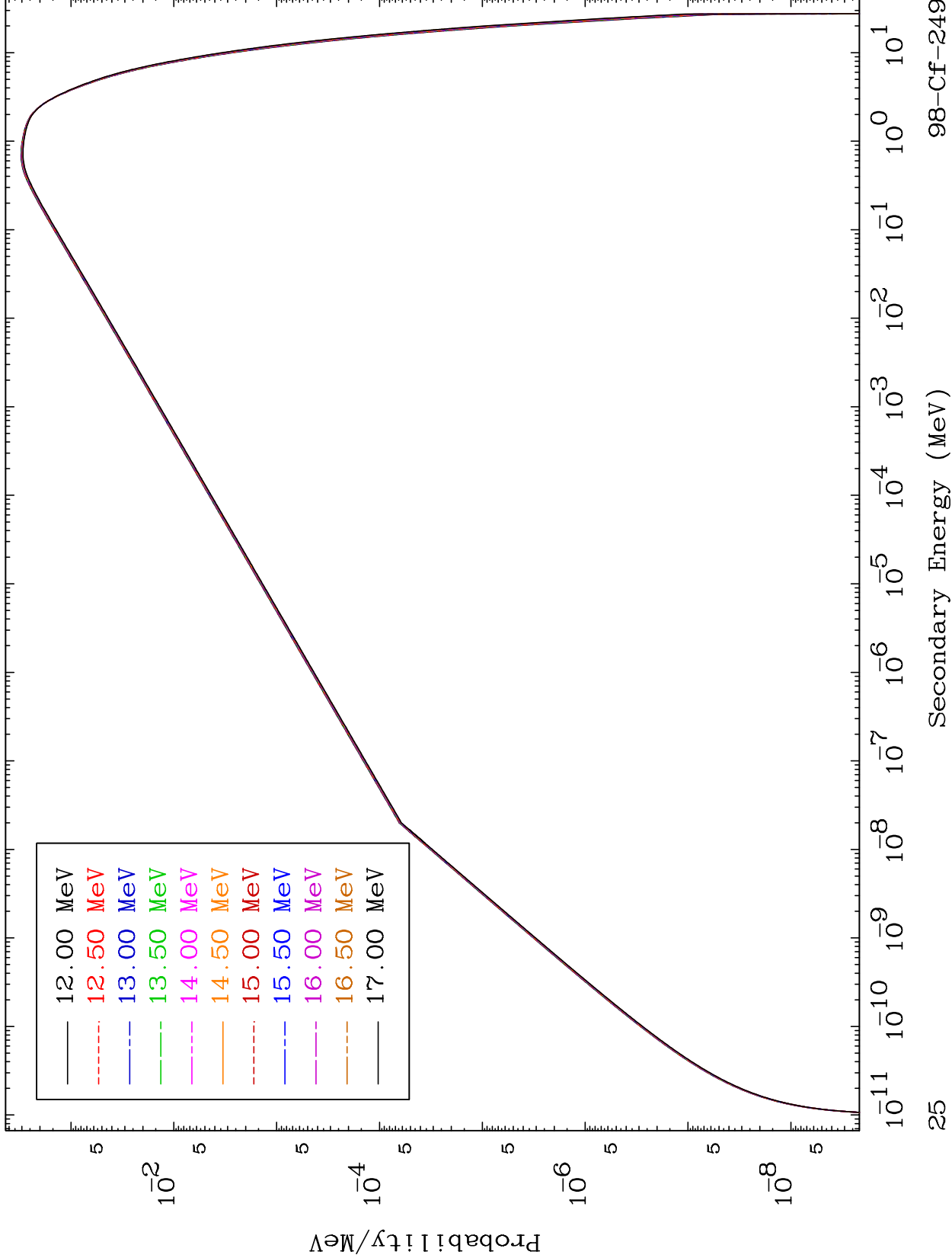
98-Cf-249



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### Fission Energy Distributions

98-Cf-249



98-Cf-249

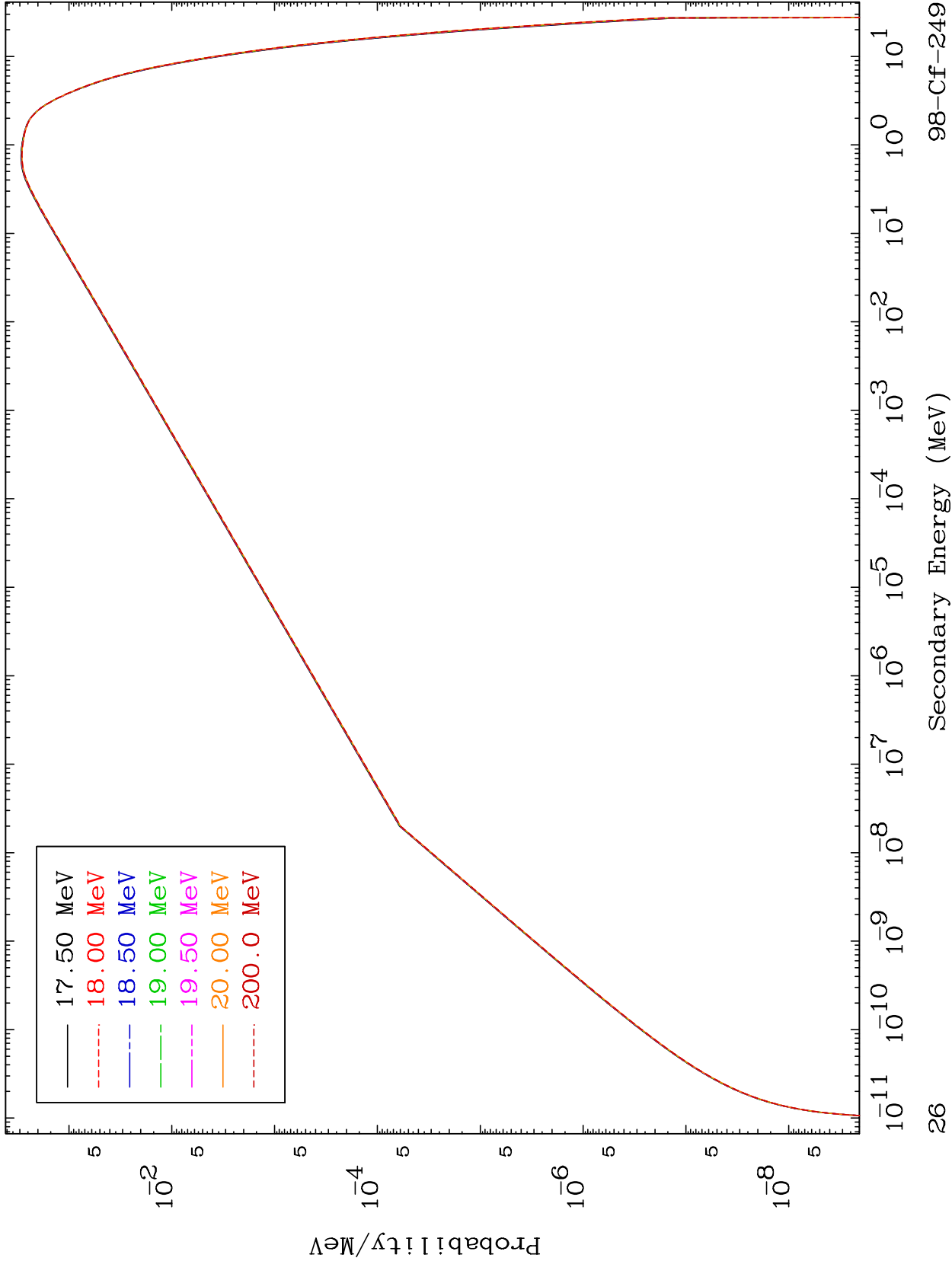
Secondary Energy (MeV)

25

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### Fission Energy Distributions

98-Cf-249

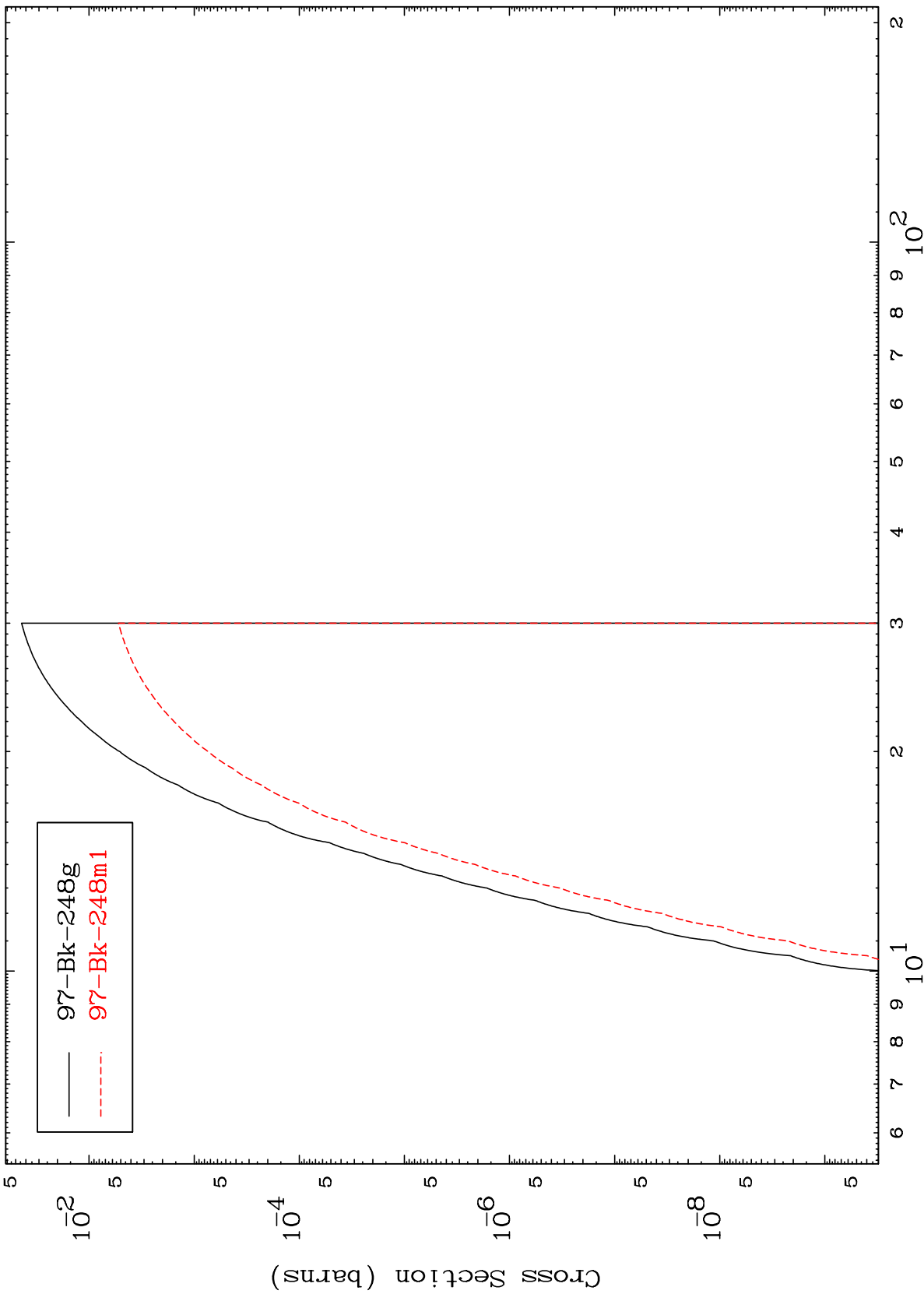


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(n,n') p

98-Cf-249

Radionuclide Production Cross Section



27

Incident Energy (MeV)

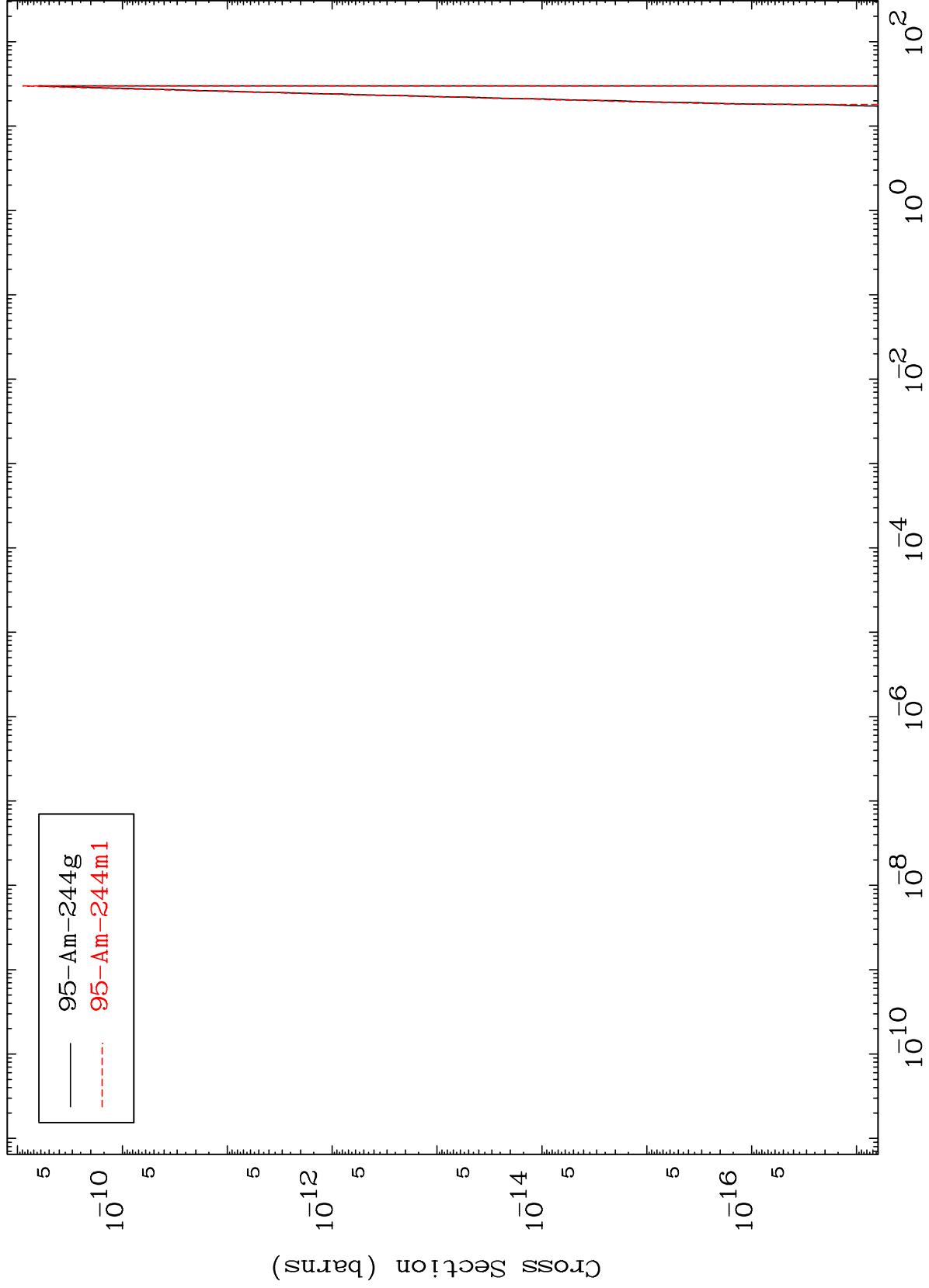
98-Cf-249

MAT 9852

(n,n') p  $\alpha$

98-Cf-249

Radionuclide Production Cross Section



28

Incident Energy (MeV)

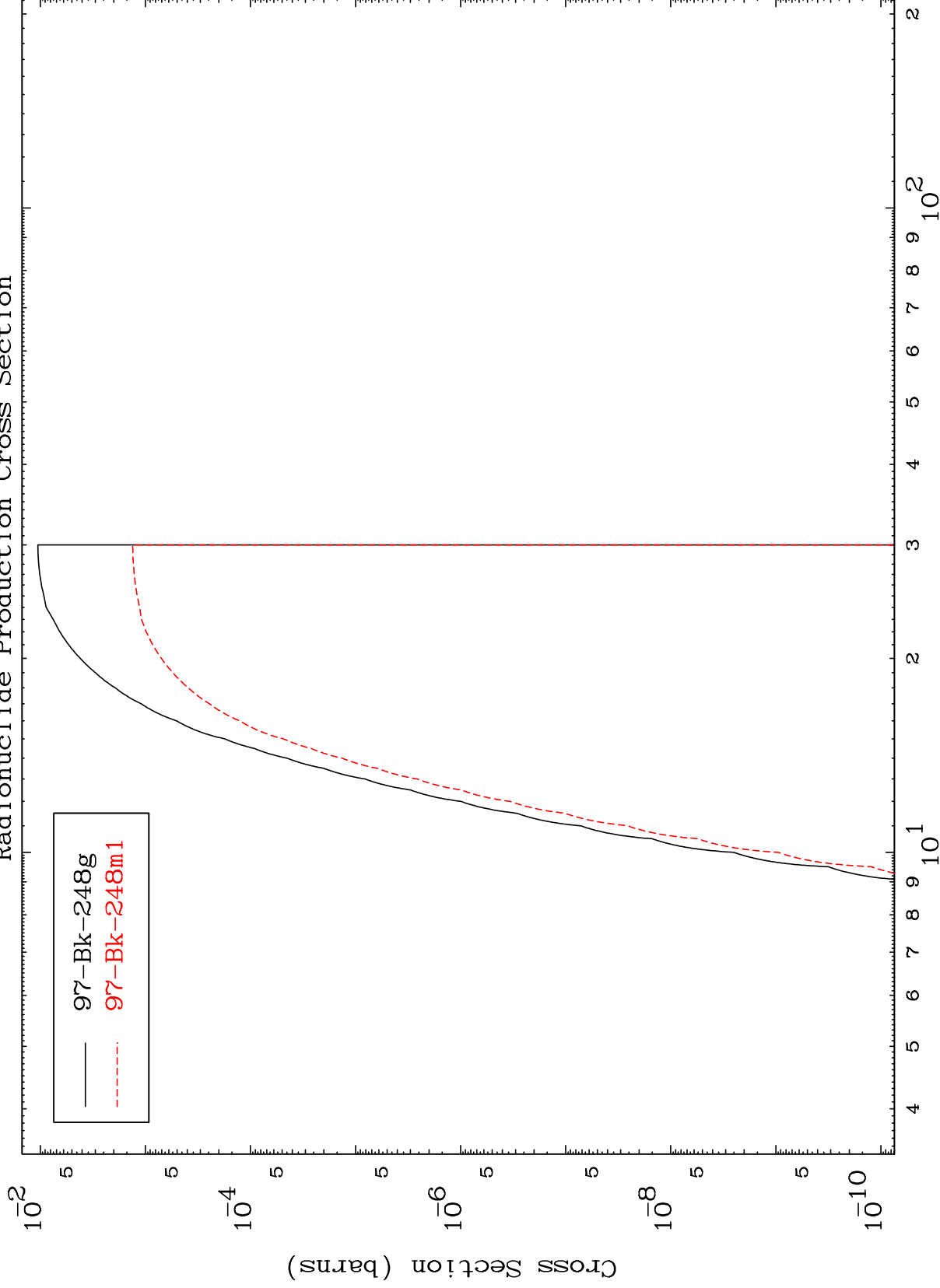
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(n,d)

98-Cf-249

Radionuclide Production Cross Section



29

Incident Energy (MeV)

98-Cf-249