

Program EVALPLOT
(Version 2021-1)

by

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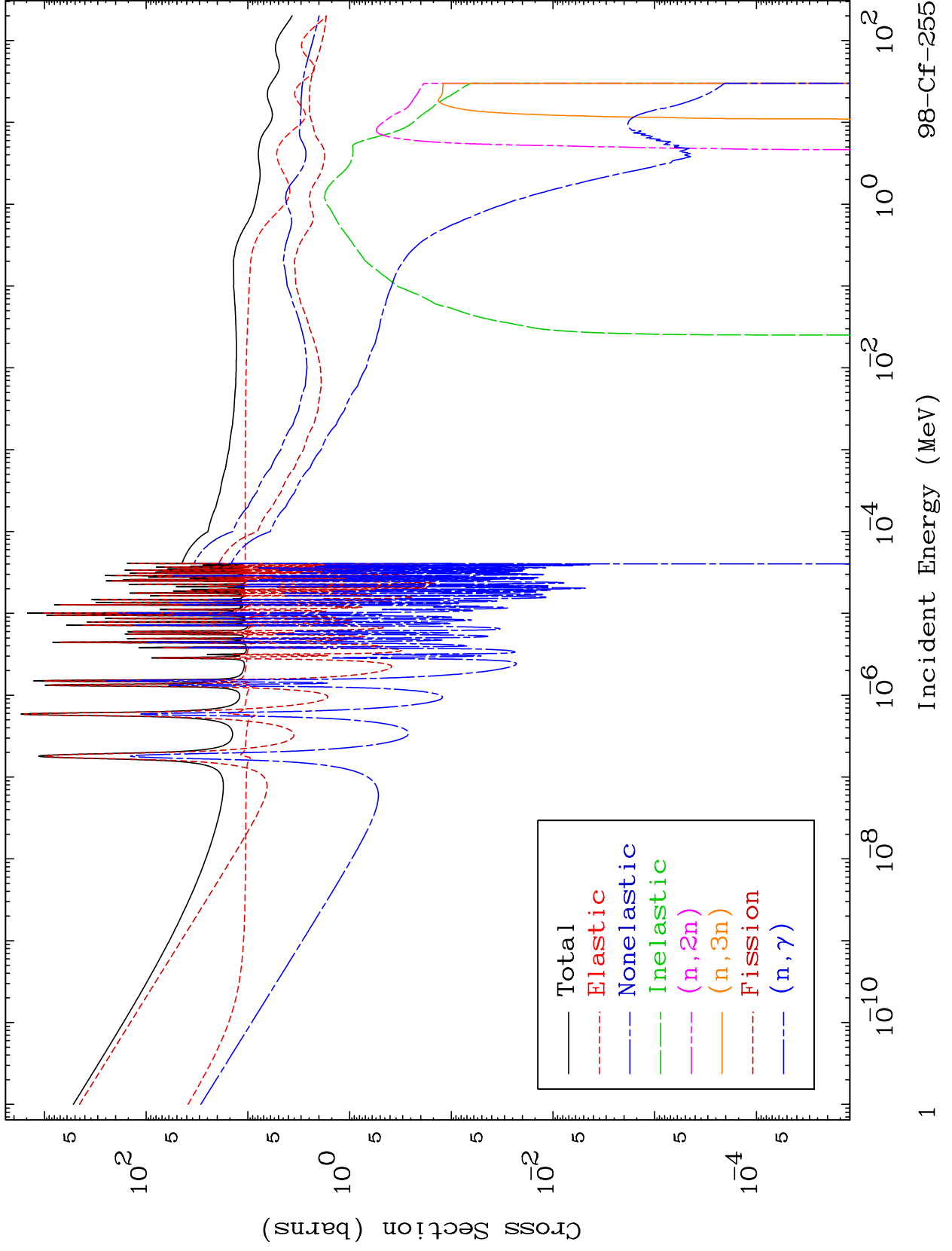
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Press Mouse Button to Start

MAT 9870

Neutron Major
293 Kelvin Cross Sections

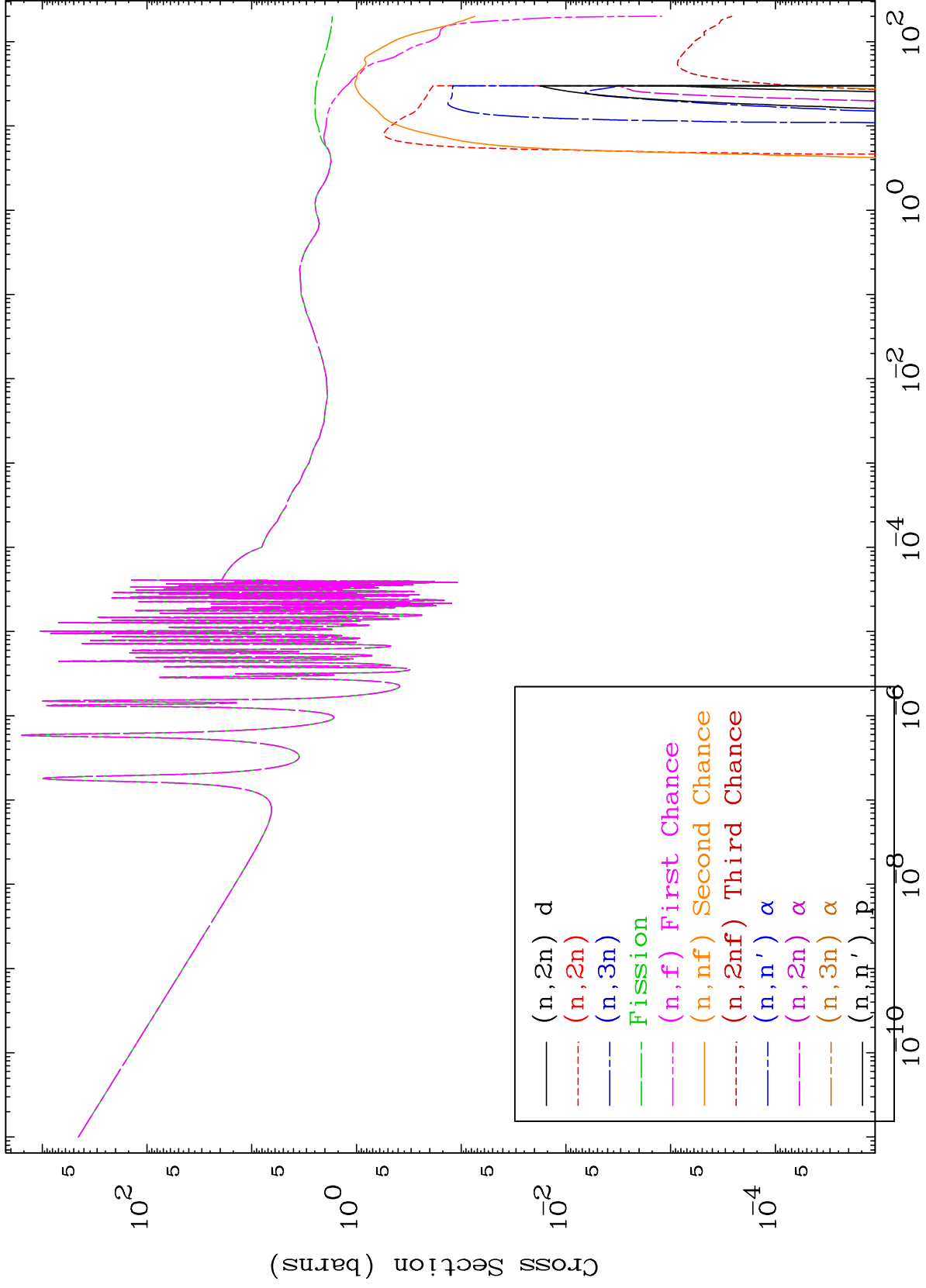
98-Cf-255



MAT 9870

Neutron Absorption
293 Kelvin Cross Sections

98-Cf-255



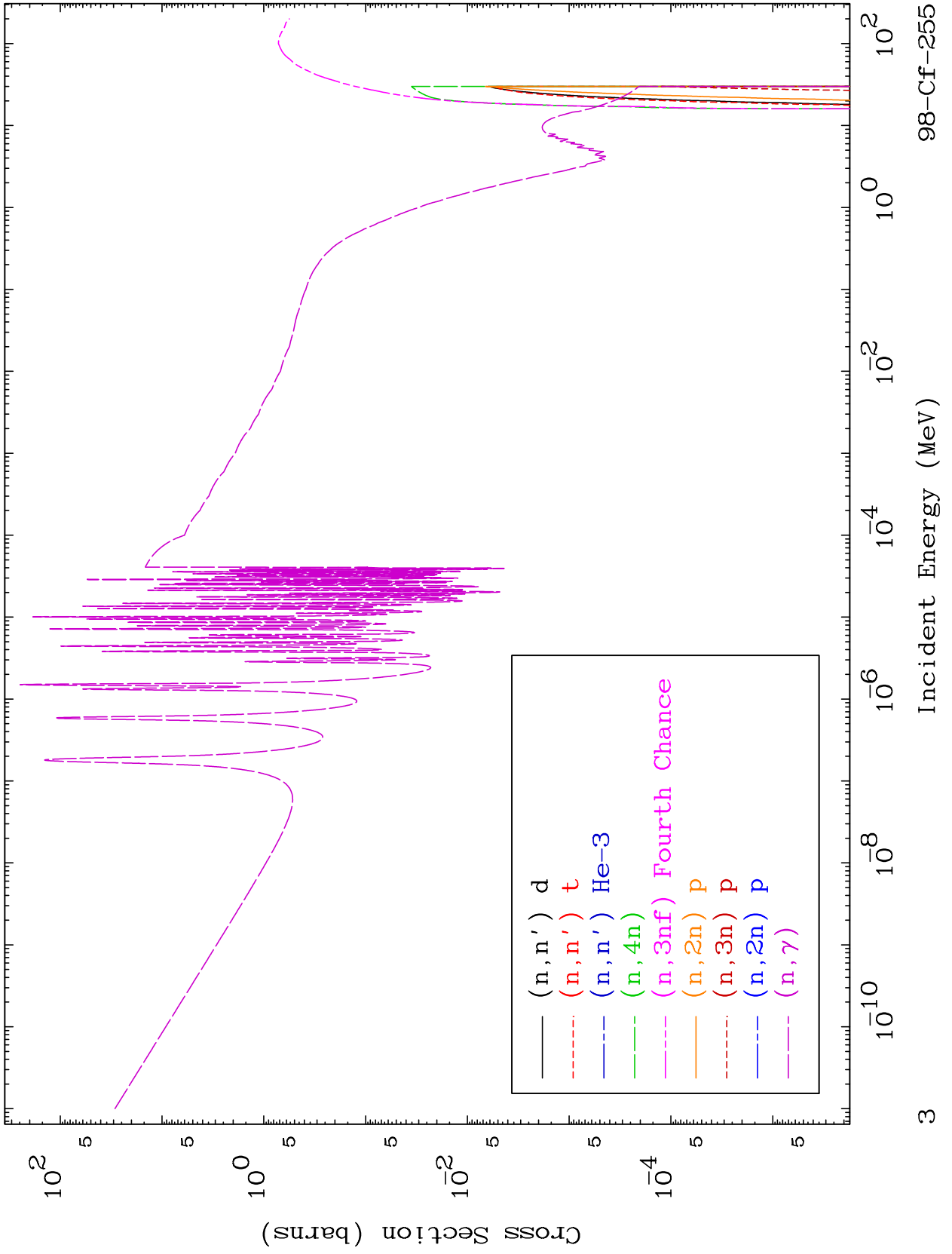
98-Cf-255

Incident Energy (MeV)

MAT 9870

Neutron Absorption
293 Kelvin Cross Sections

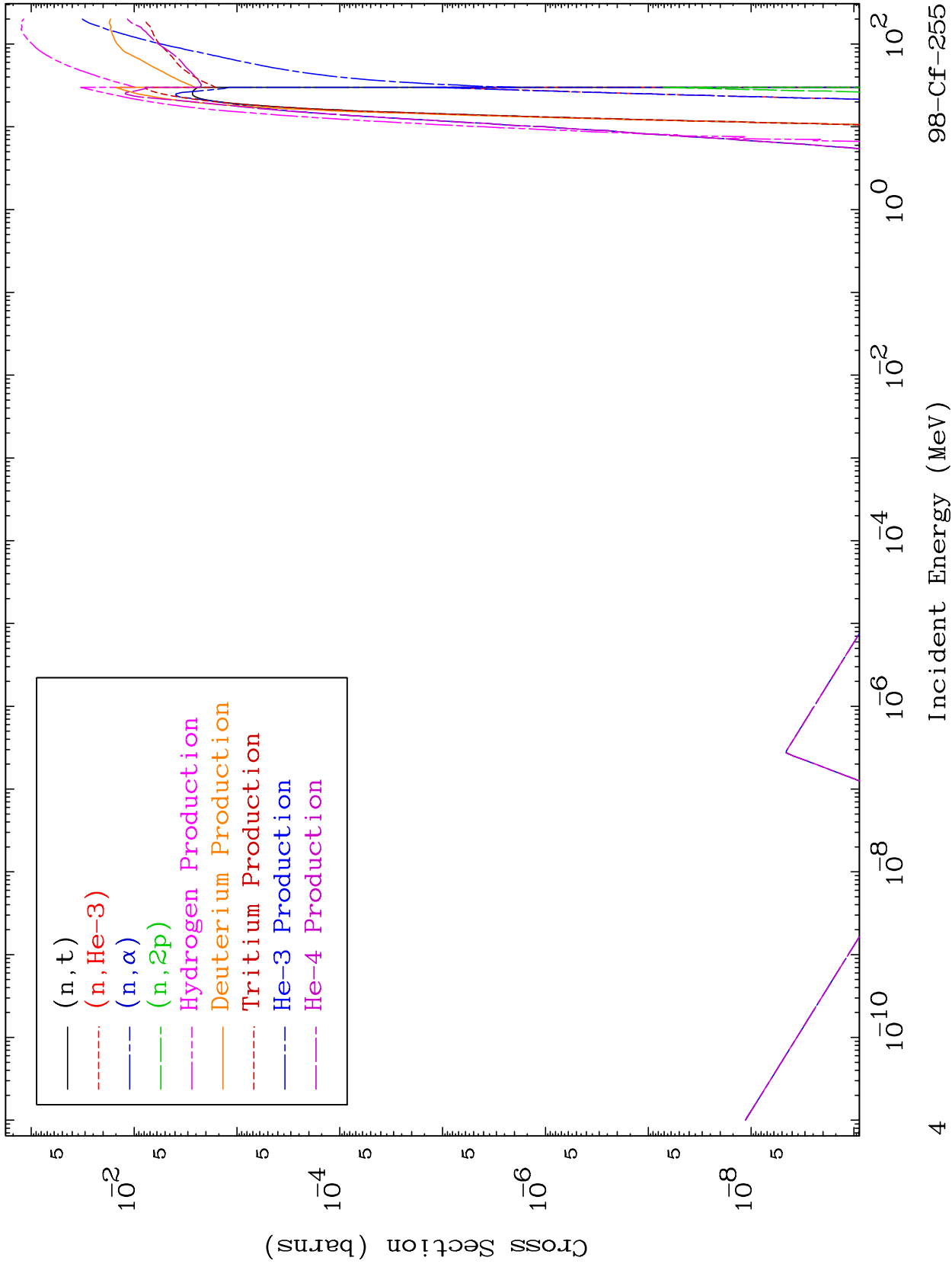
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Neutron Absorption
293 Kelvin Cross Sections

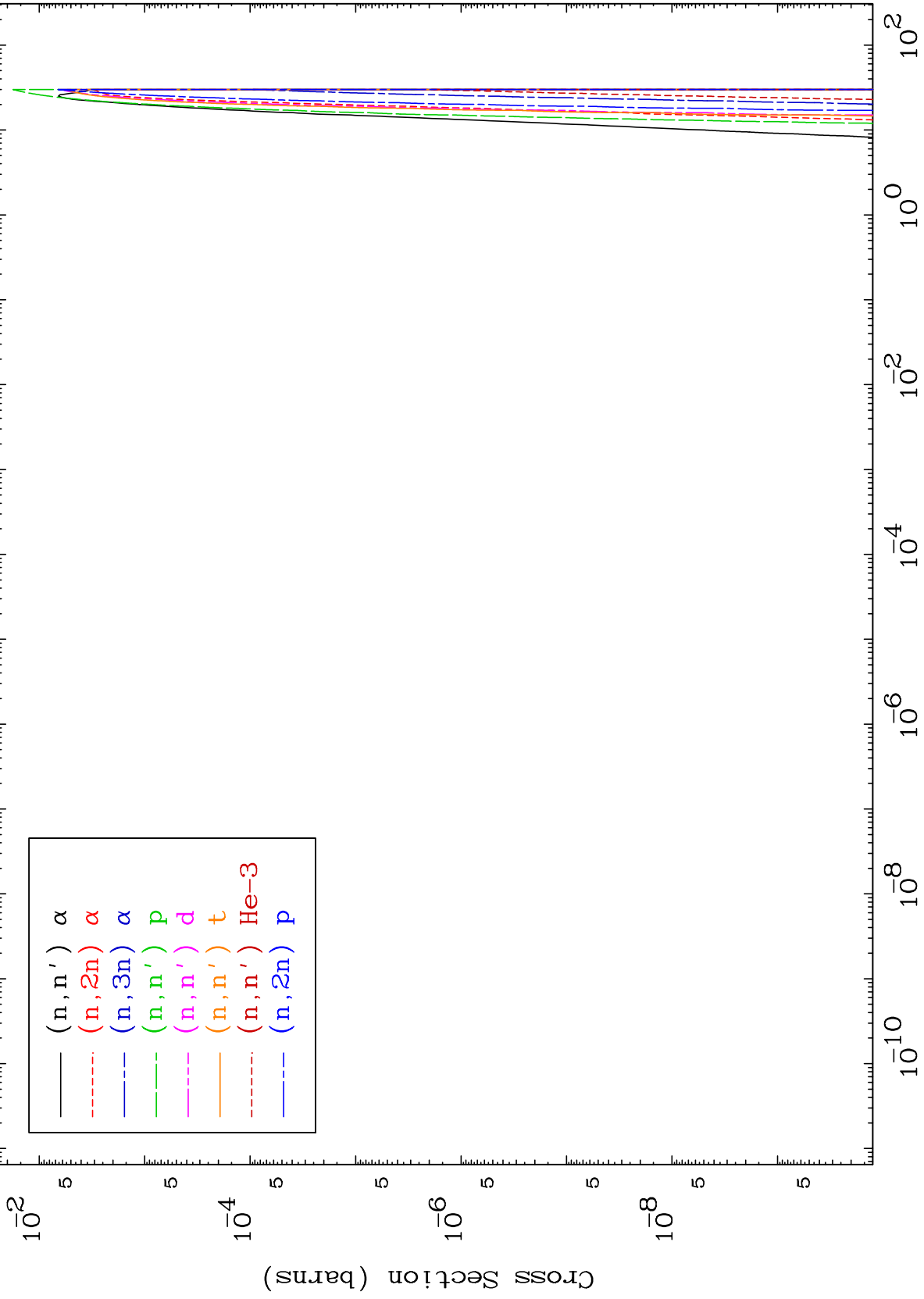
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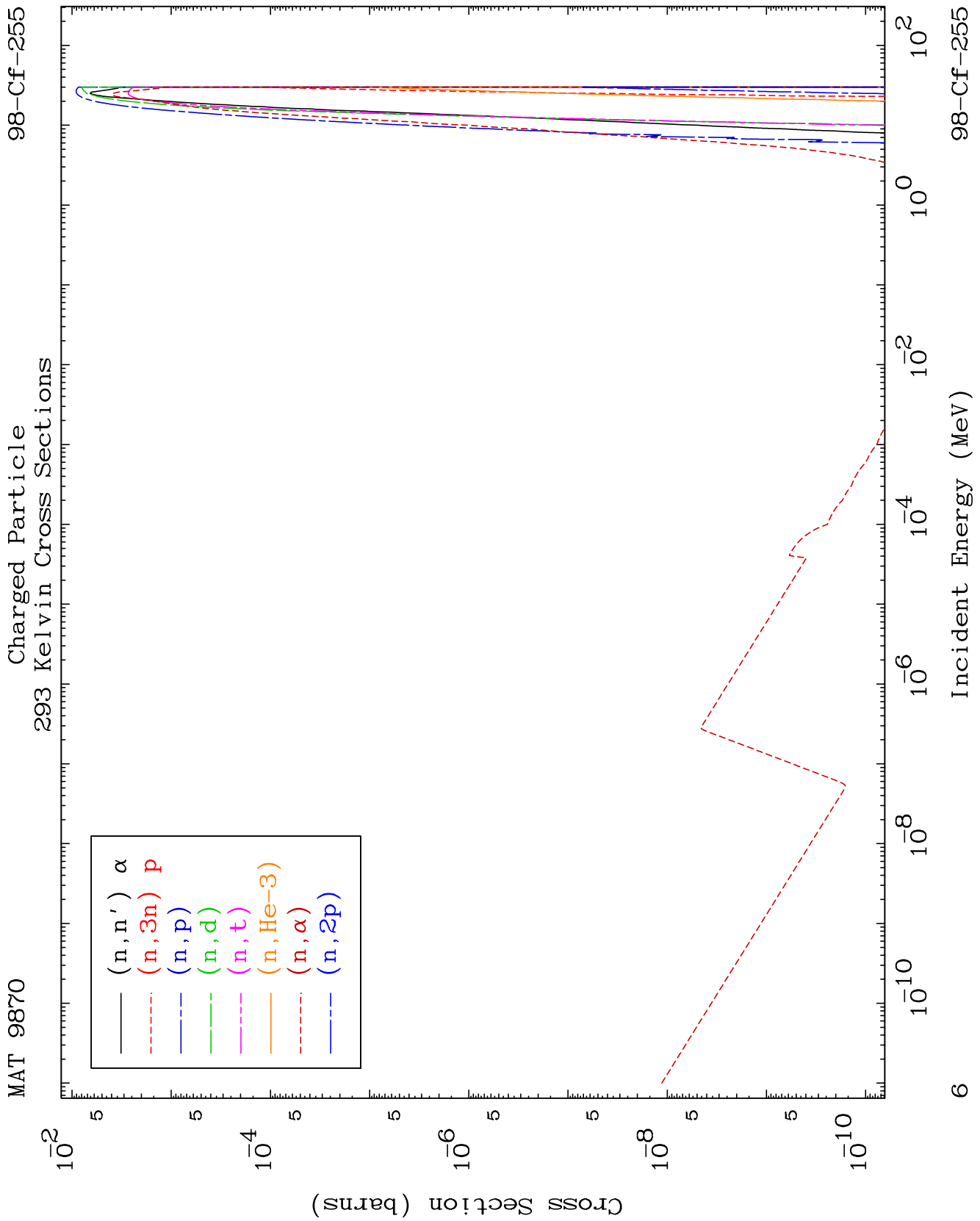


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Charged Particle
293 Kelvin Cross Sections

98-Cf-255

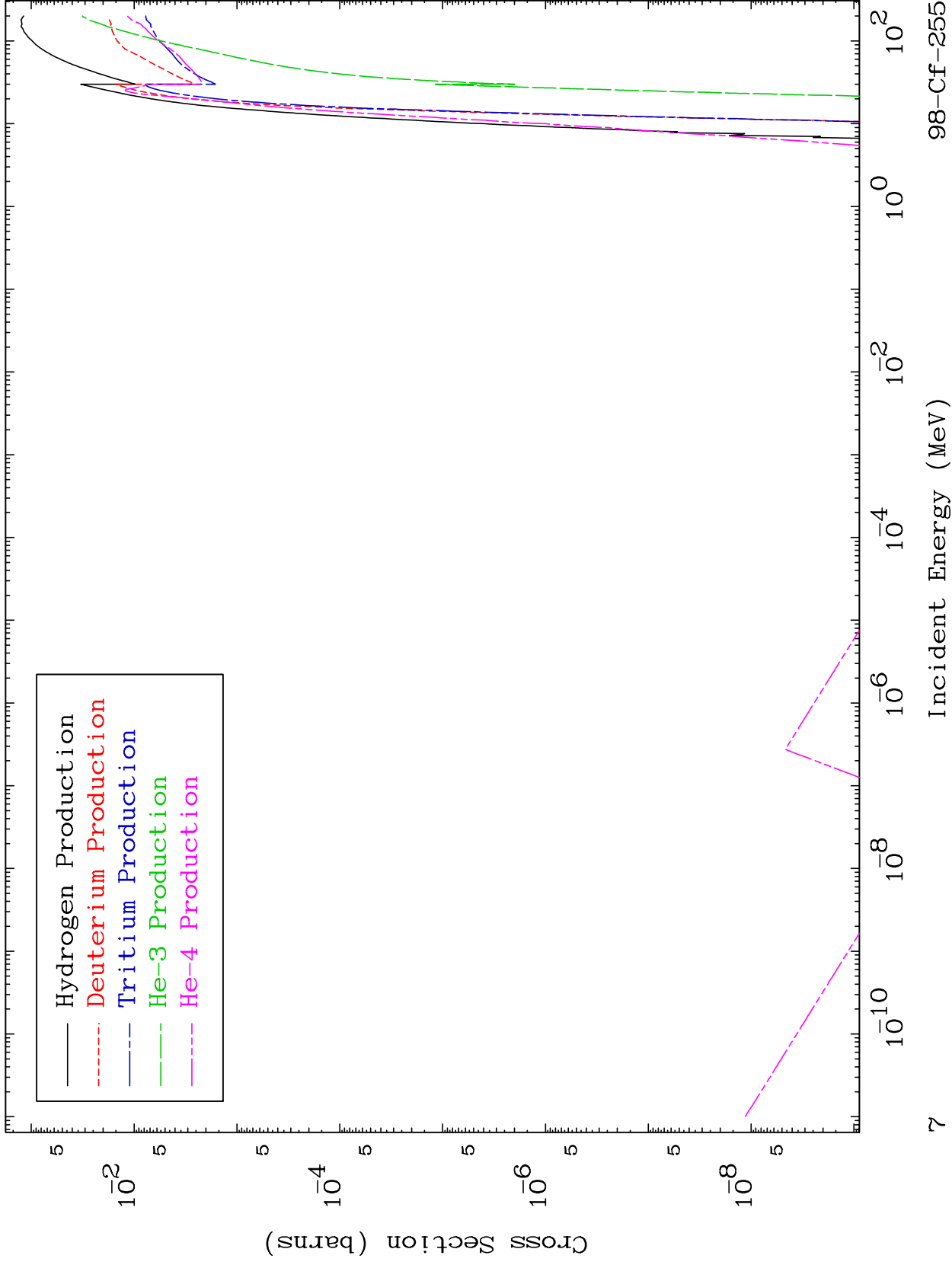




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Particle Production
293 Kelvin Cross Sections

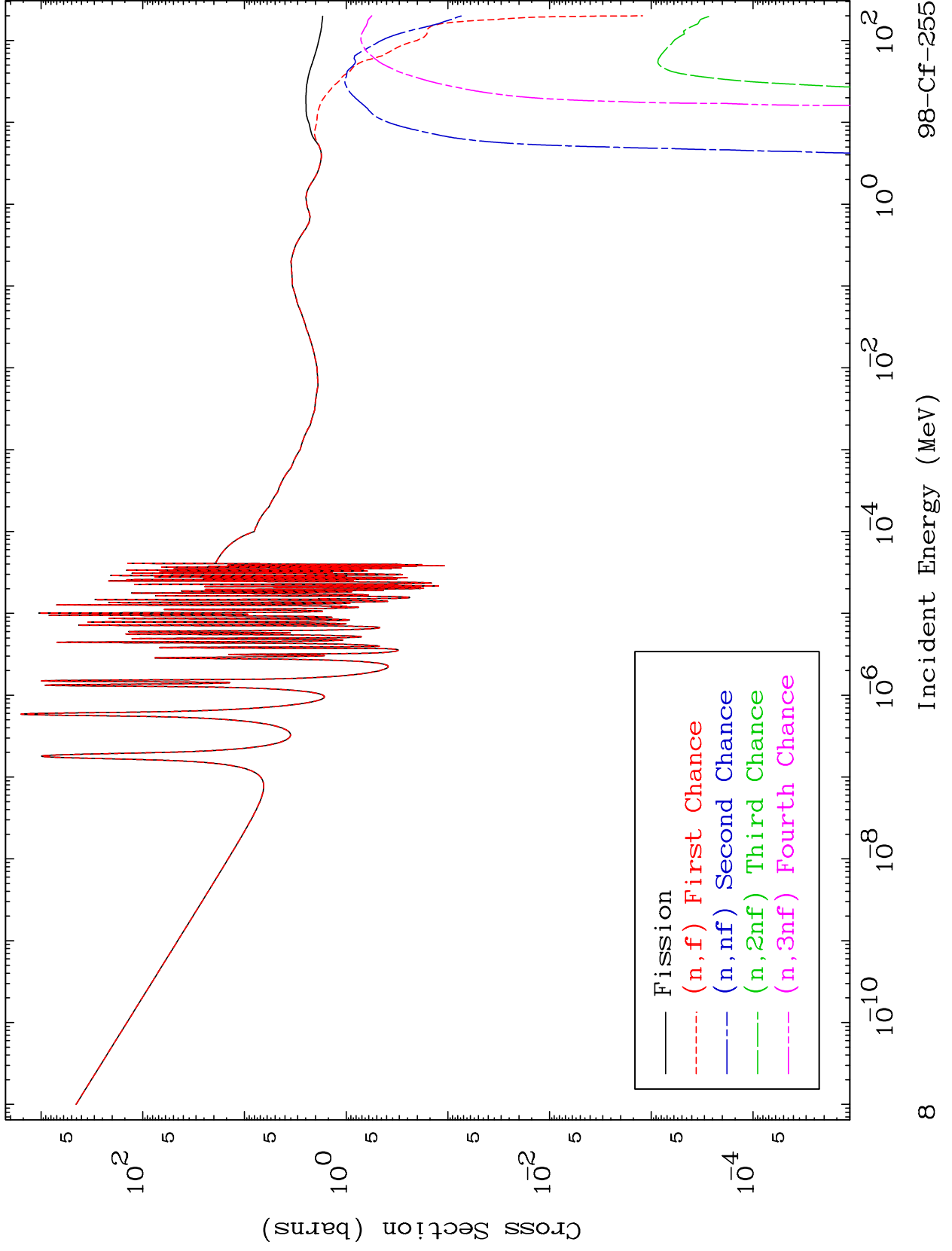
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MAT 9870

Fission
293 Kelvin Cross Sections

98-Cf-255

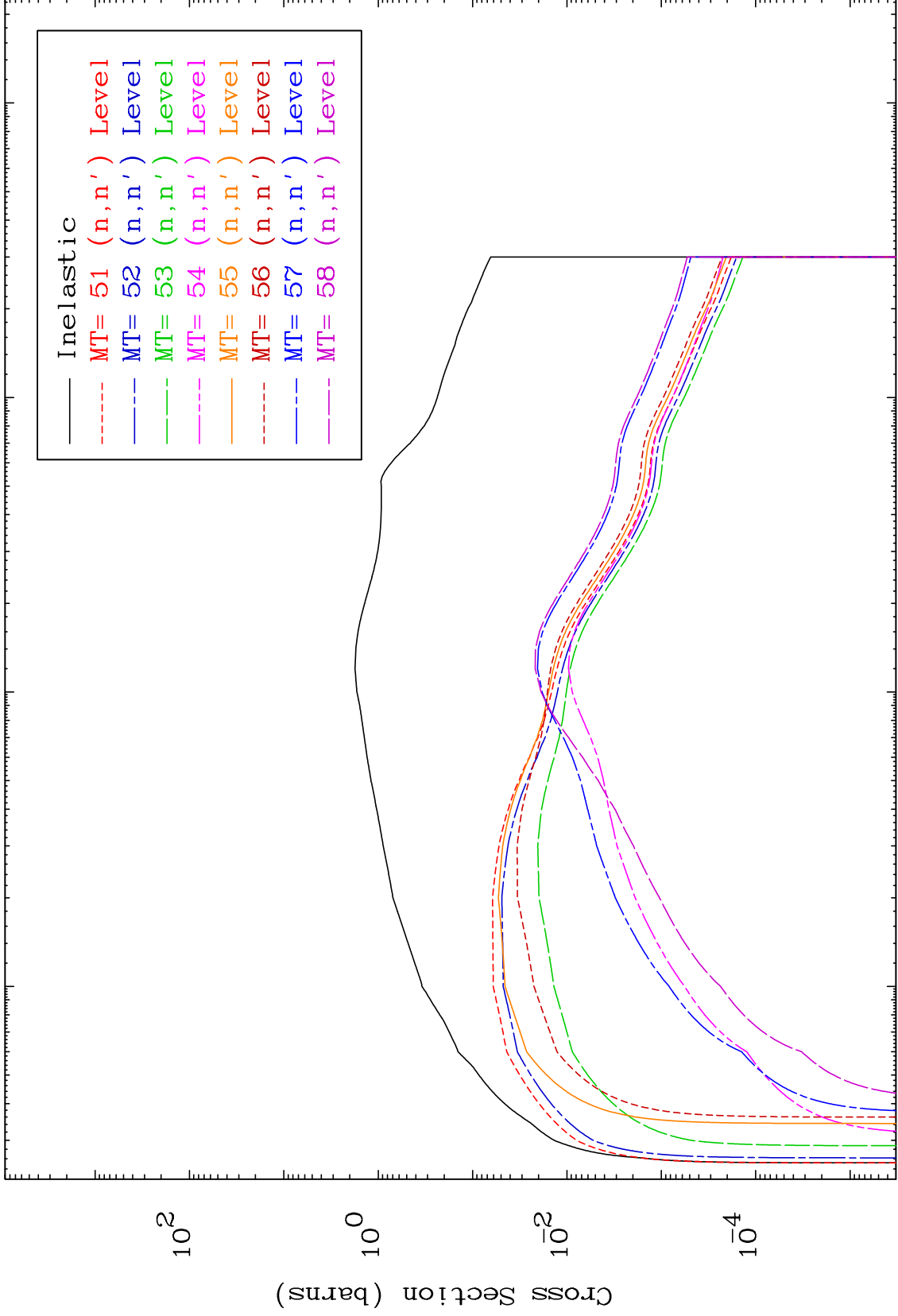


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(n,n') Levels

98-Cf-255

293 Kelvin Cross Sections

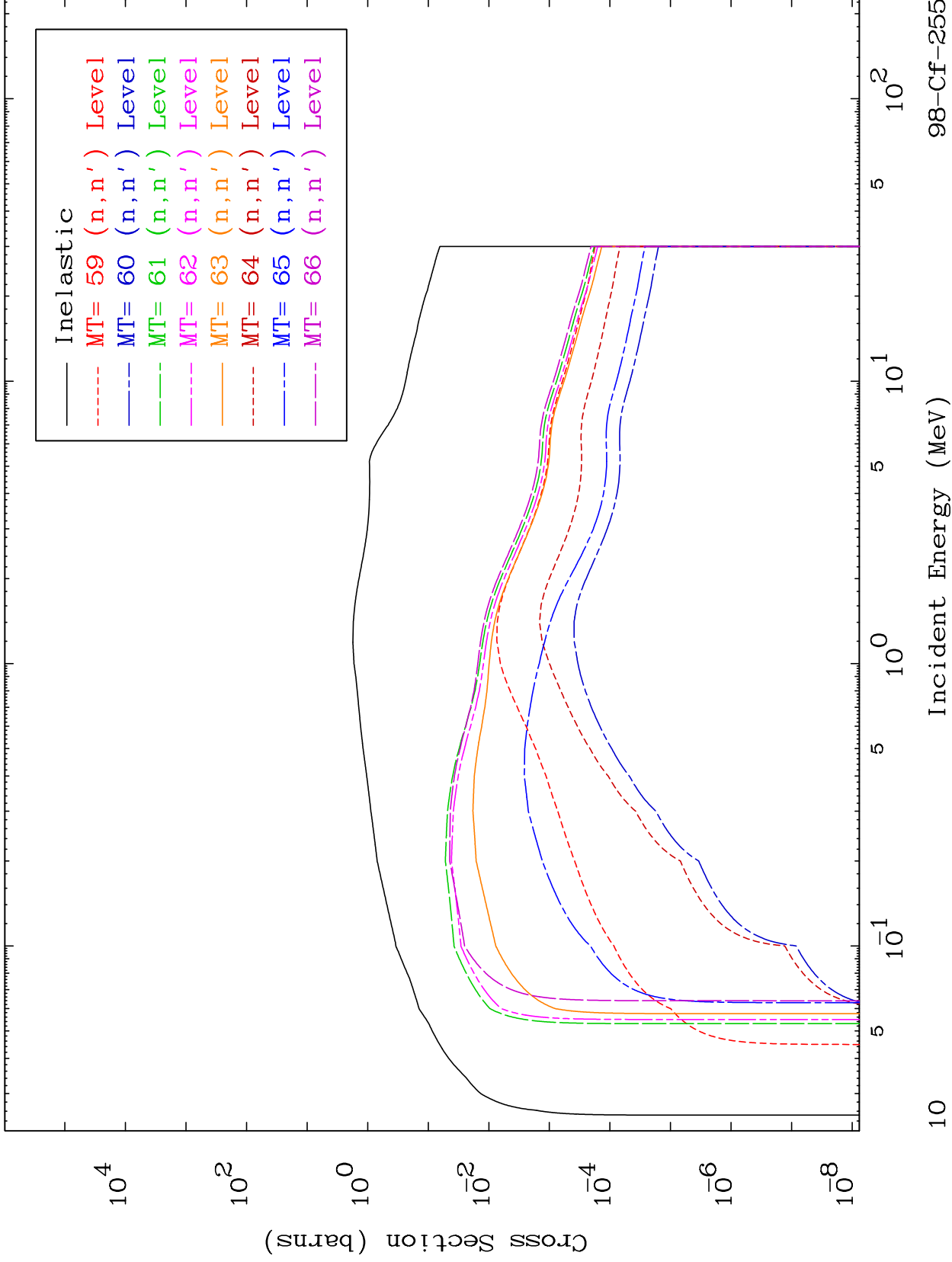


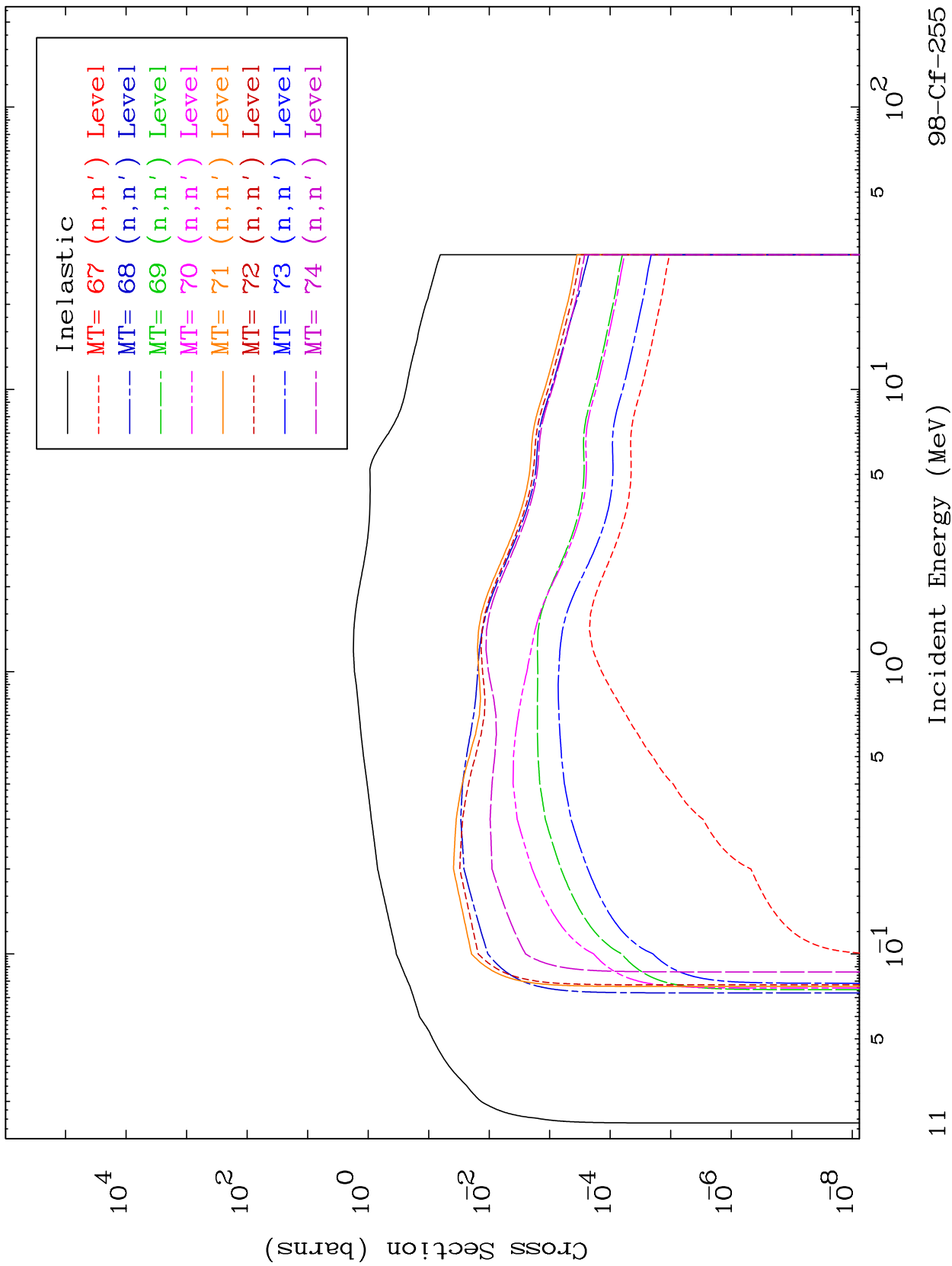
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(n,n') Levels

98-Cf-255

293 Kelvin Cross Sections

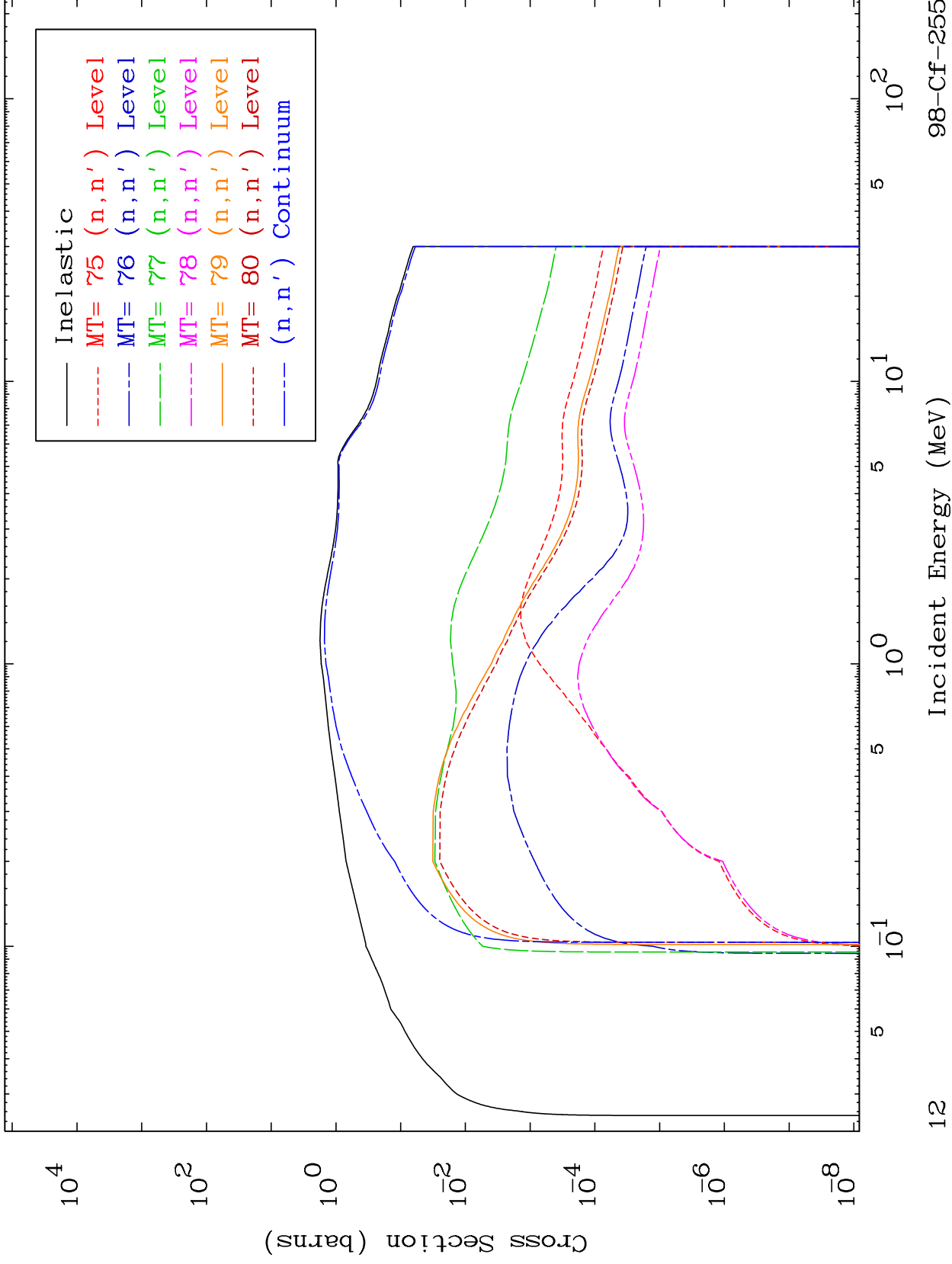




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(n,n') Levels
293 Kelvin Cross Sections

98-Cf-255



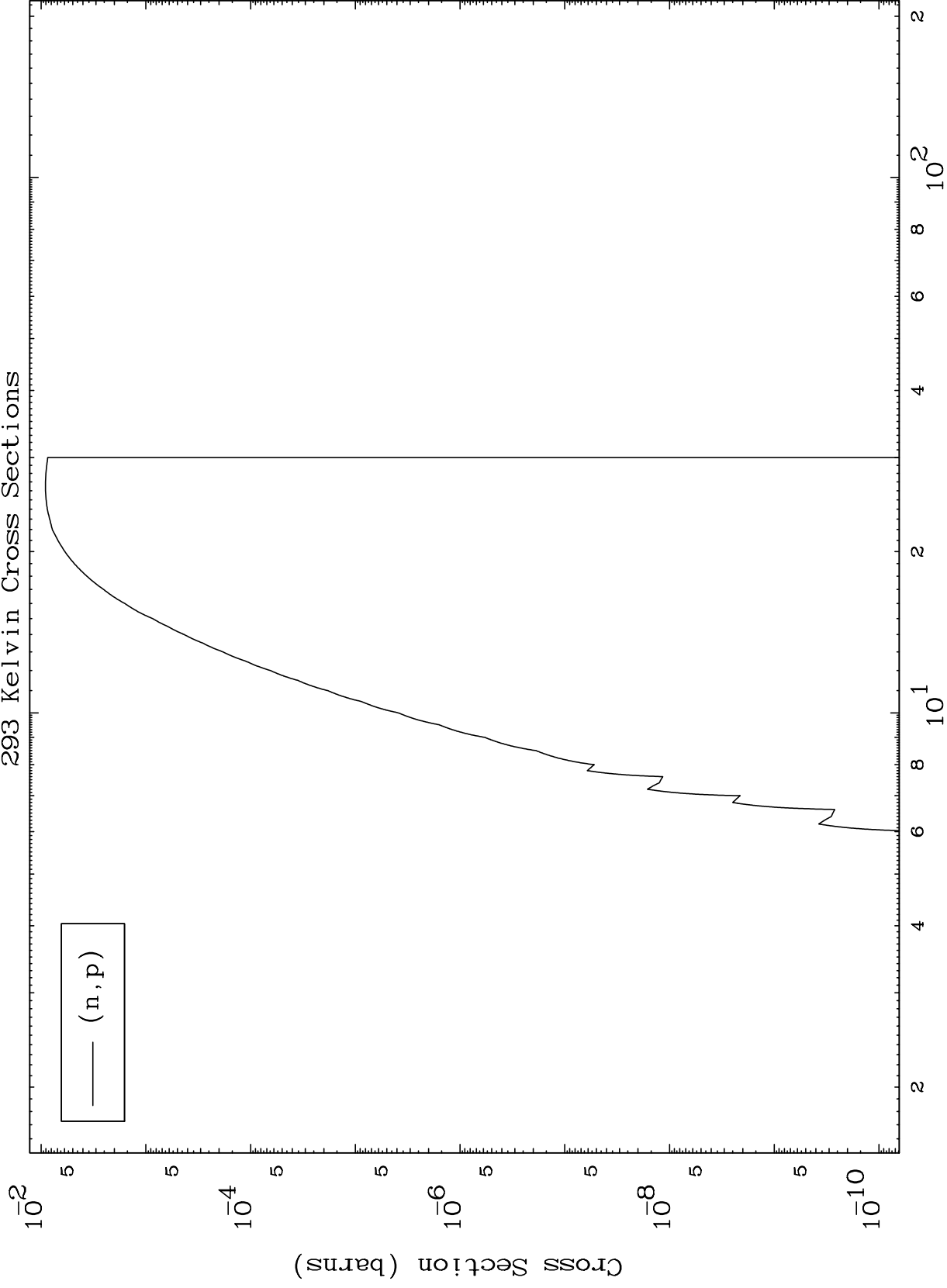
12

98-Cf-255

MAT 9870

(n,p) Levels
293 Kelvin Cross Sections

98-Cf-255



13

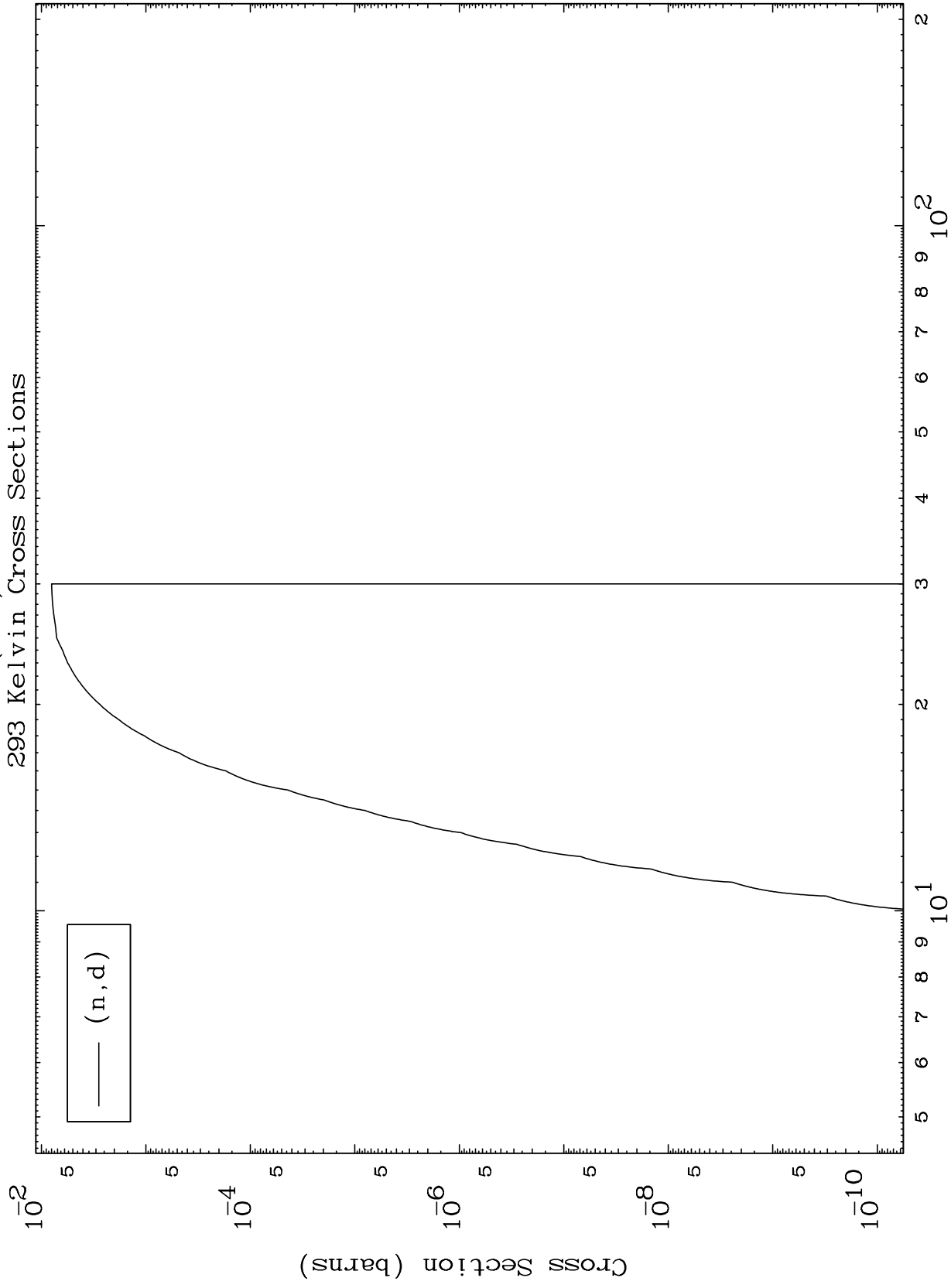
Incident Energy (MeV)

98-Cf-255

MAT 9870

(n,d) Levels
293 Kelvin Cross Sections

98-Cf-255



14

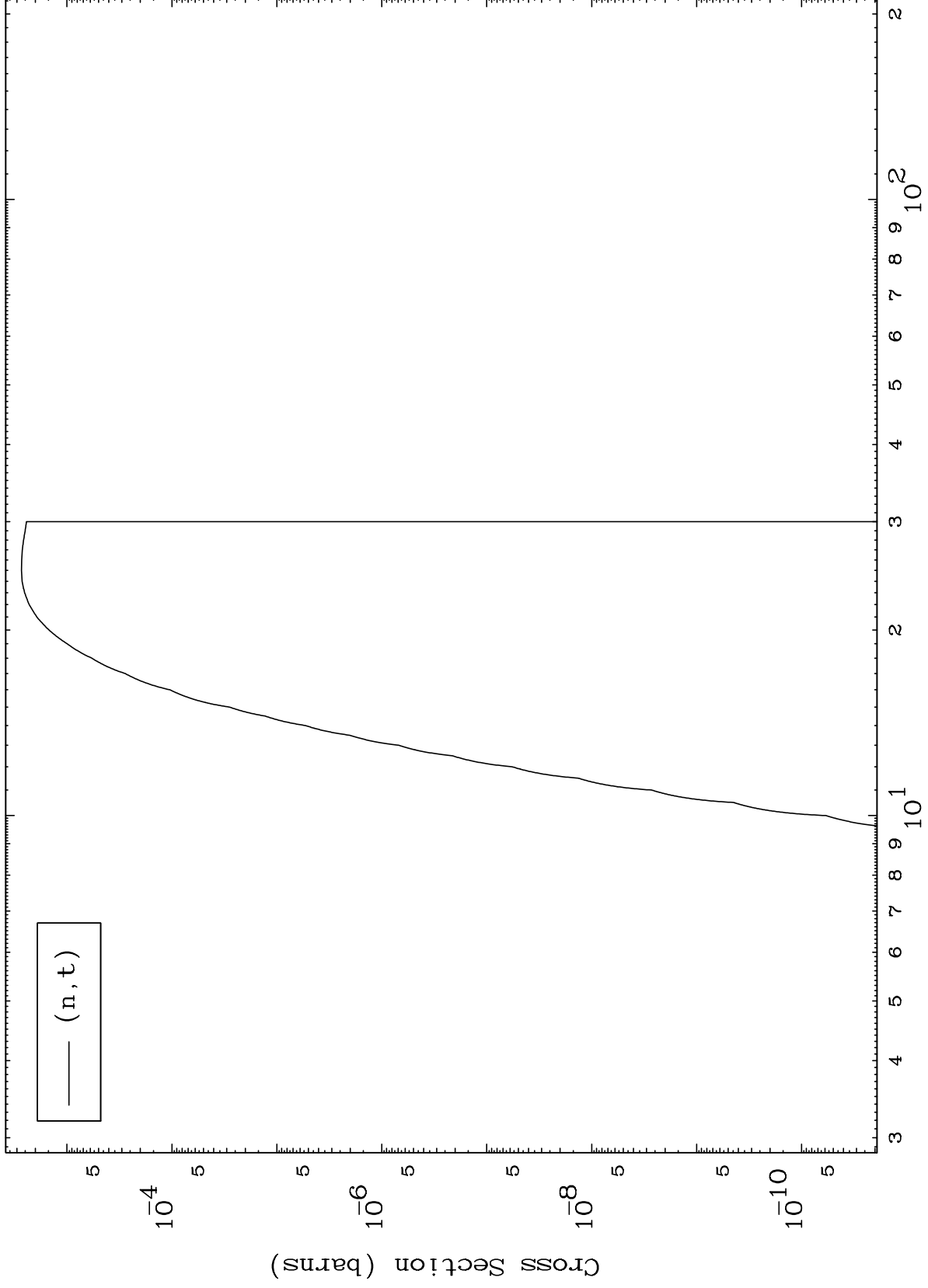
Incident Energy (MeV)

98-Cf-255

MAT 9870

(n,t) Levels
293 Kelvin Cross Sections

98-Cf-255



(n,t)

15

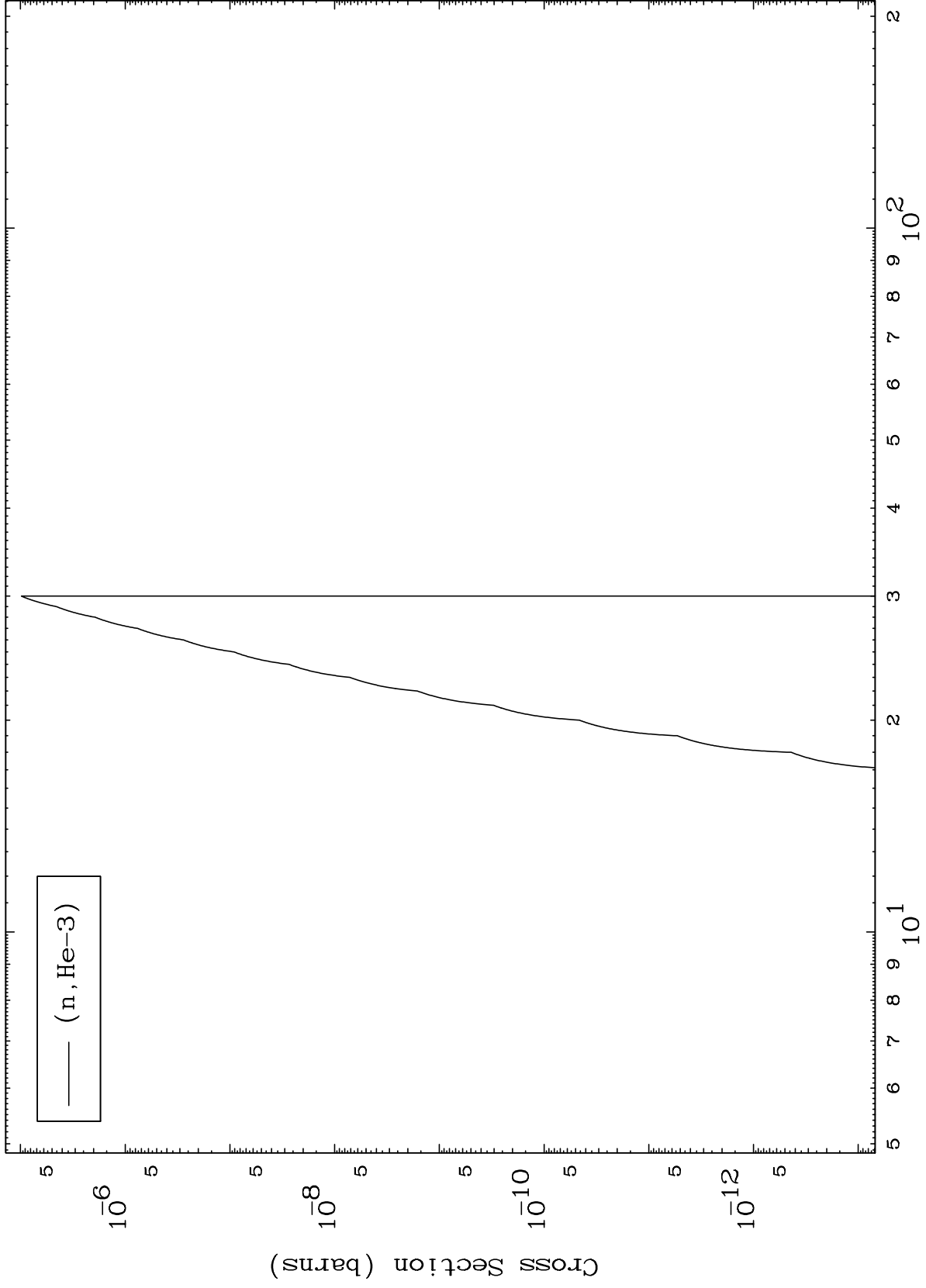
Incident Energy (MeV)

98-Cf-255

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(n,He3) Levels
293 Kelvin Cross Sections

98-Cf-255



16

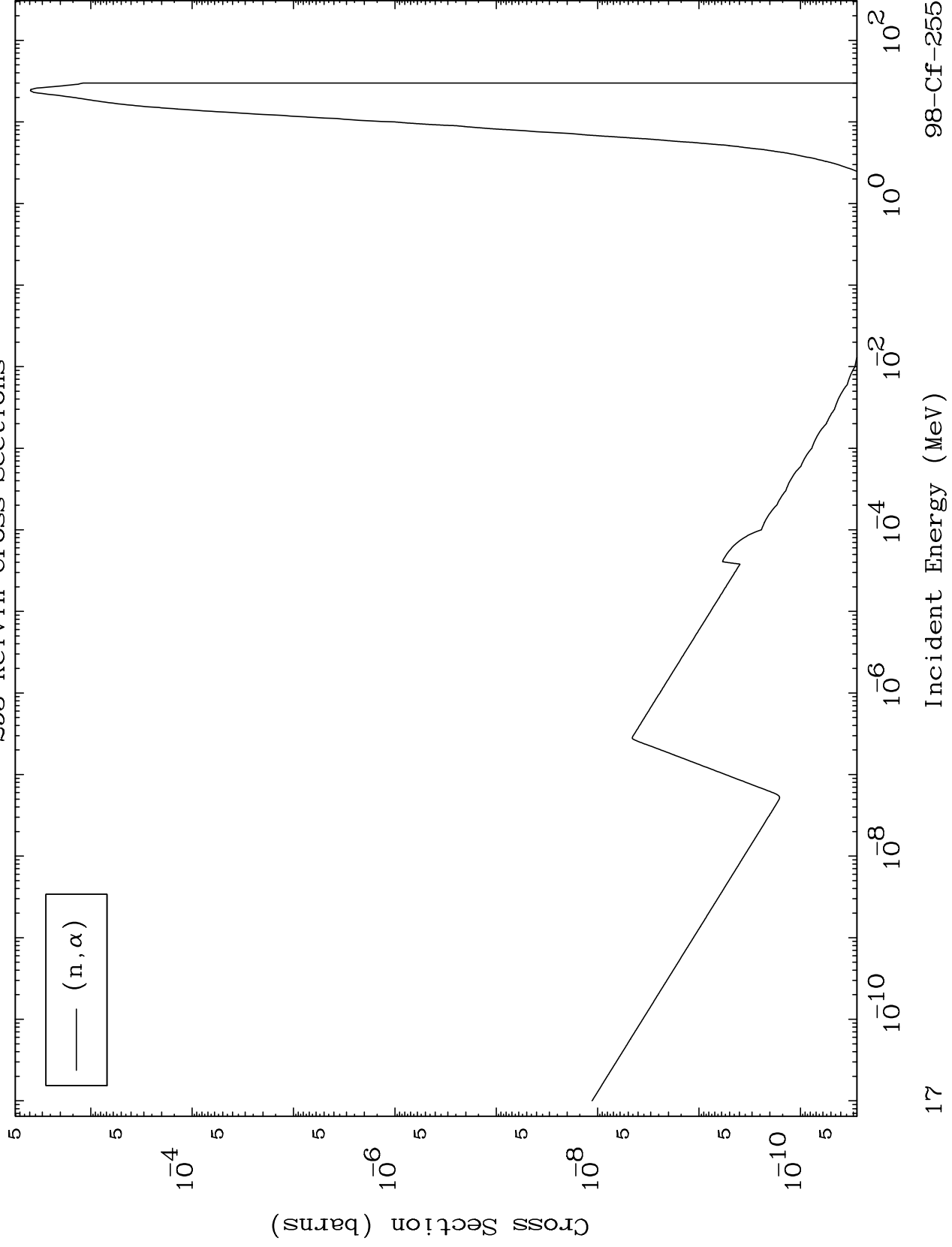
Incident Energy (MeV)

98-Cf-255

MAT 9870

(n, α) Levels
293 Kelvin Cross Sections

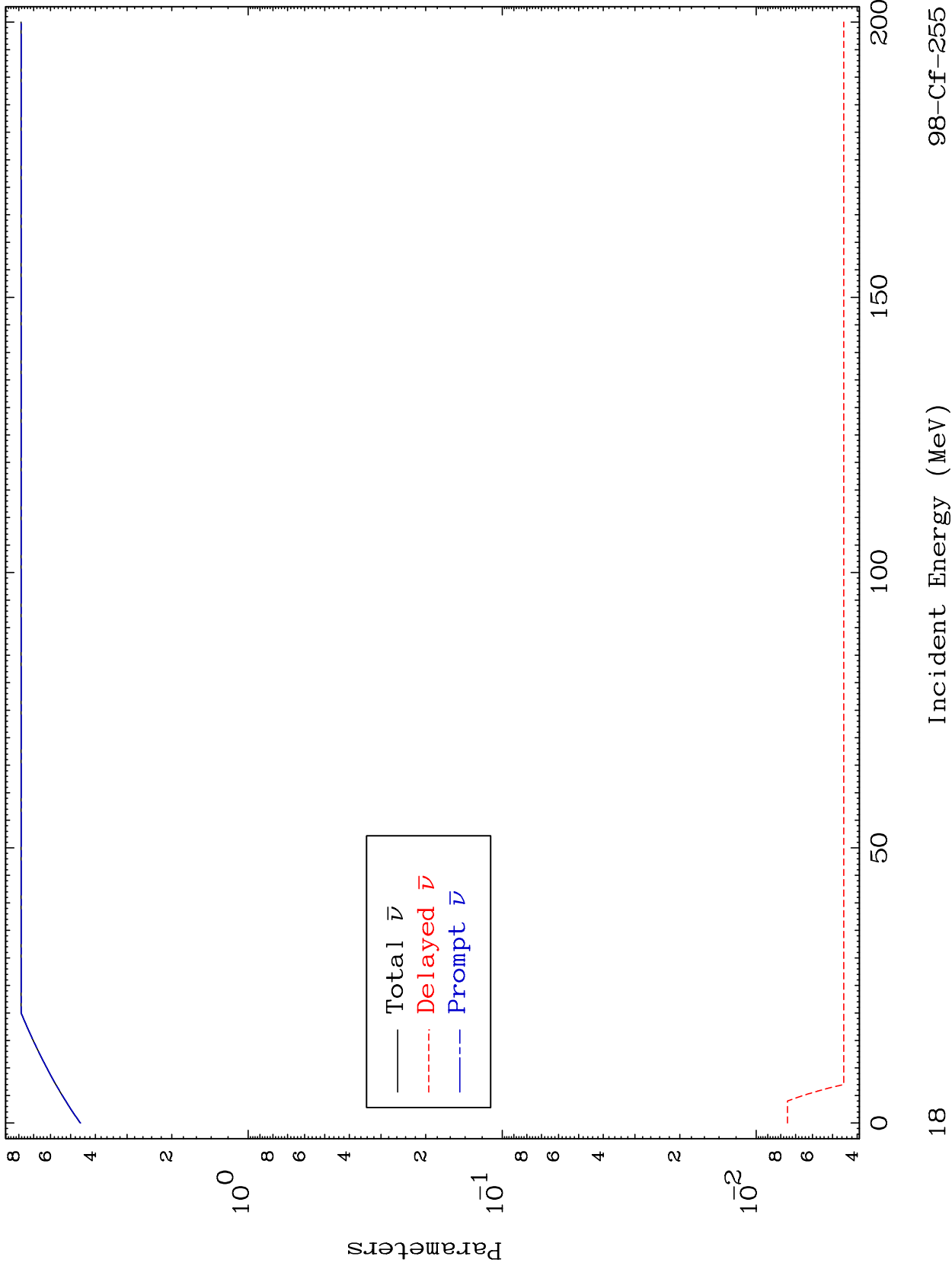
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Energy Release
Parameters

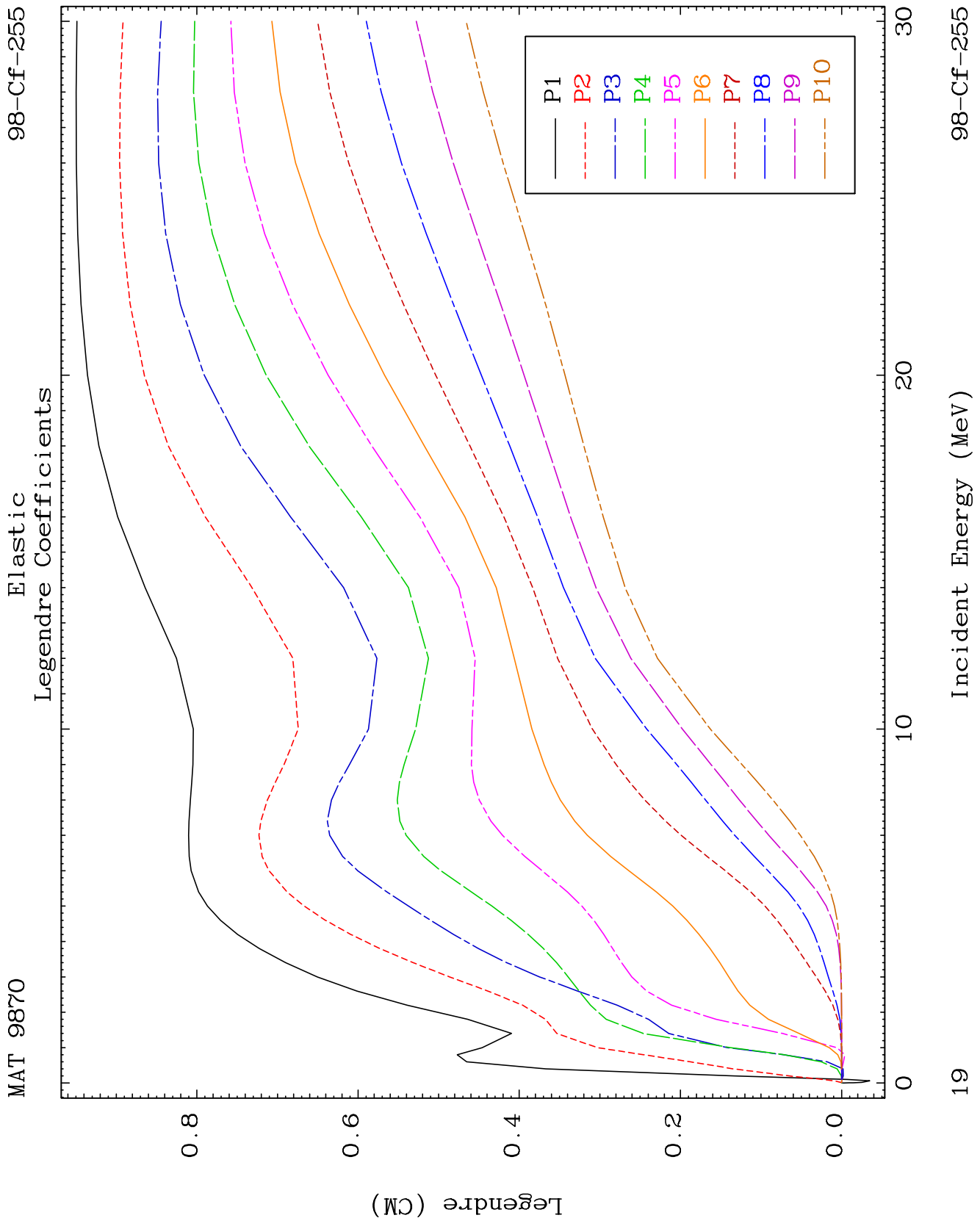
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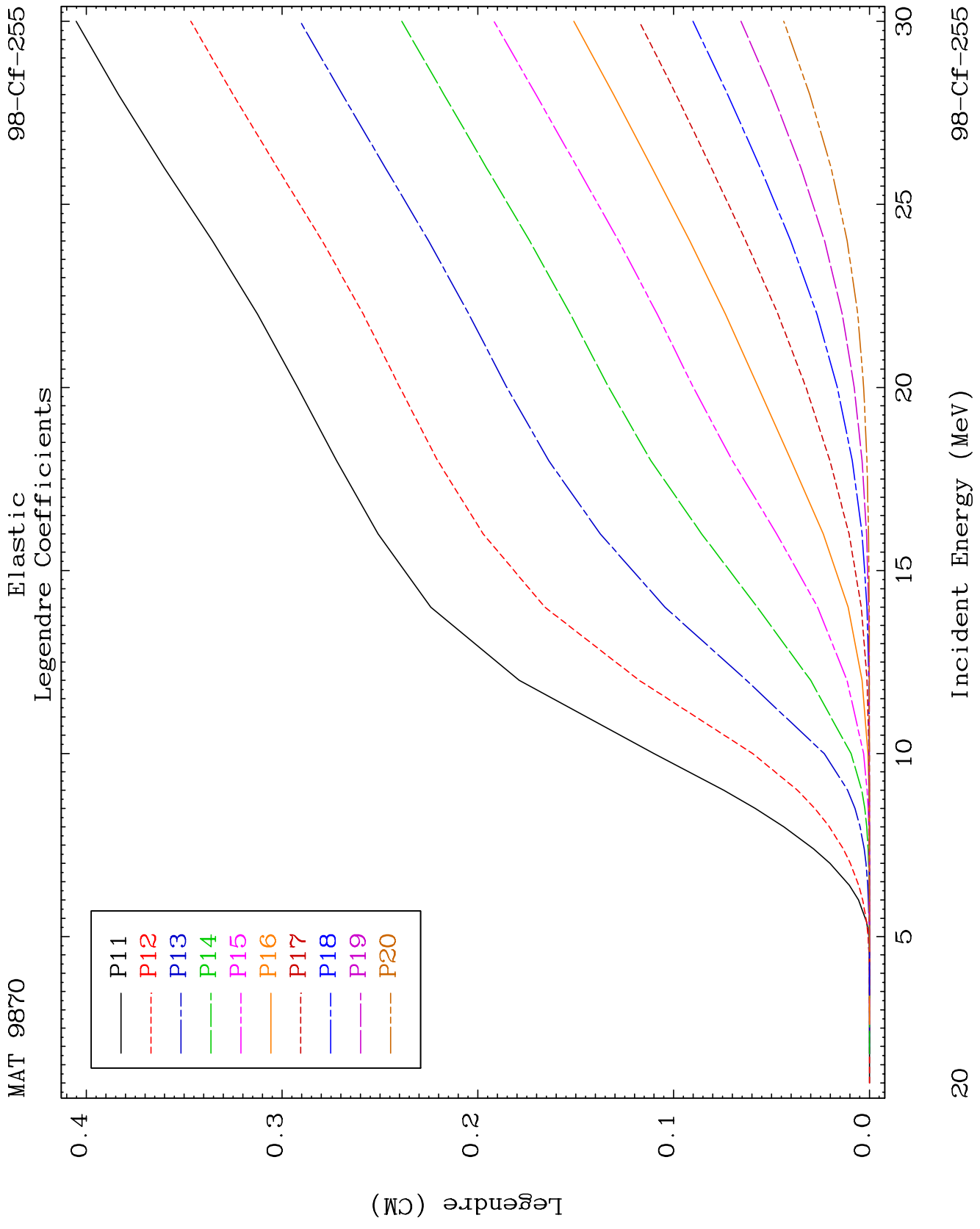


18

Incident Energy (MeV)

98-Cf-255

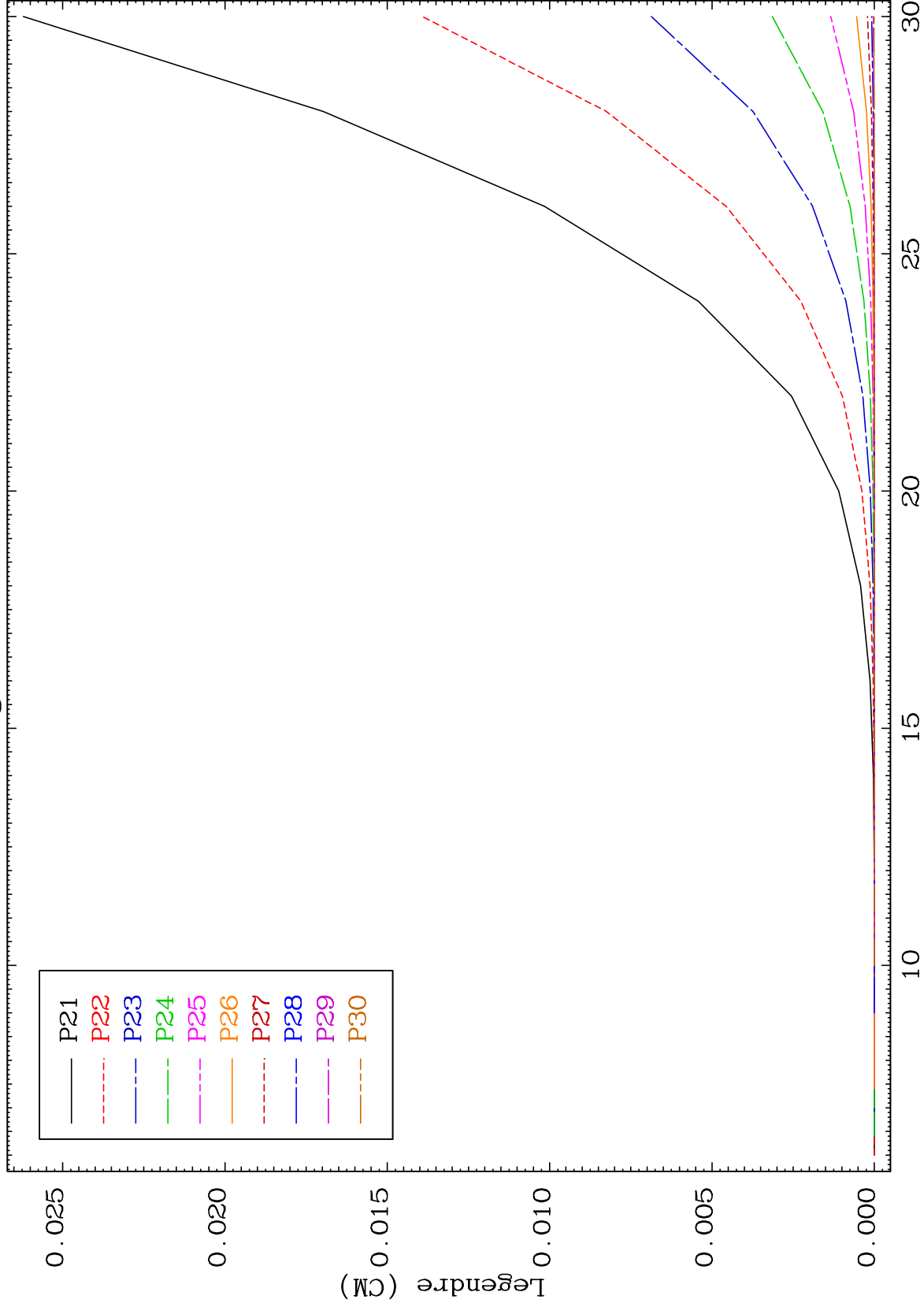




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Elastic Legendre Coefficients

98-Cf-255



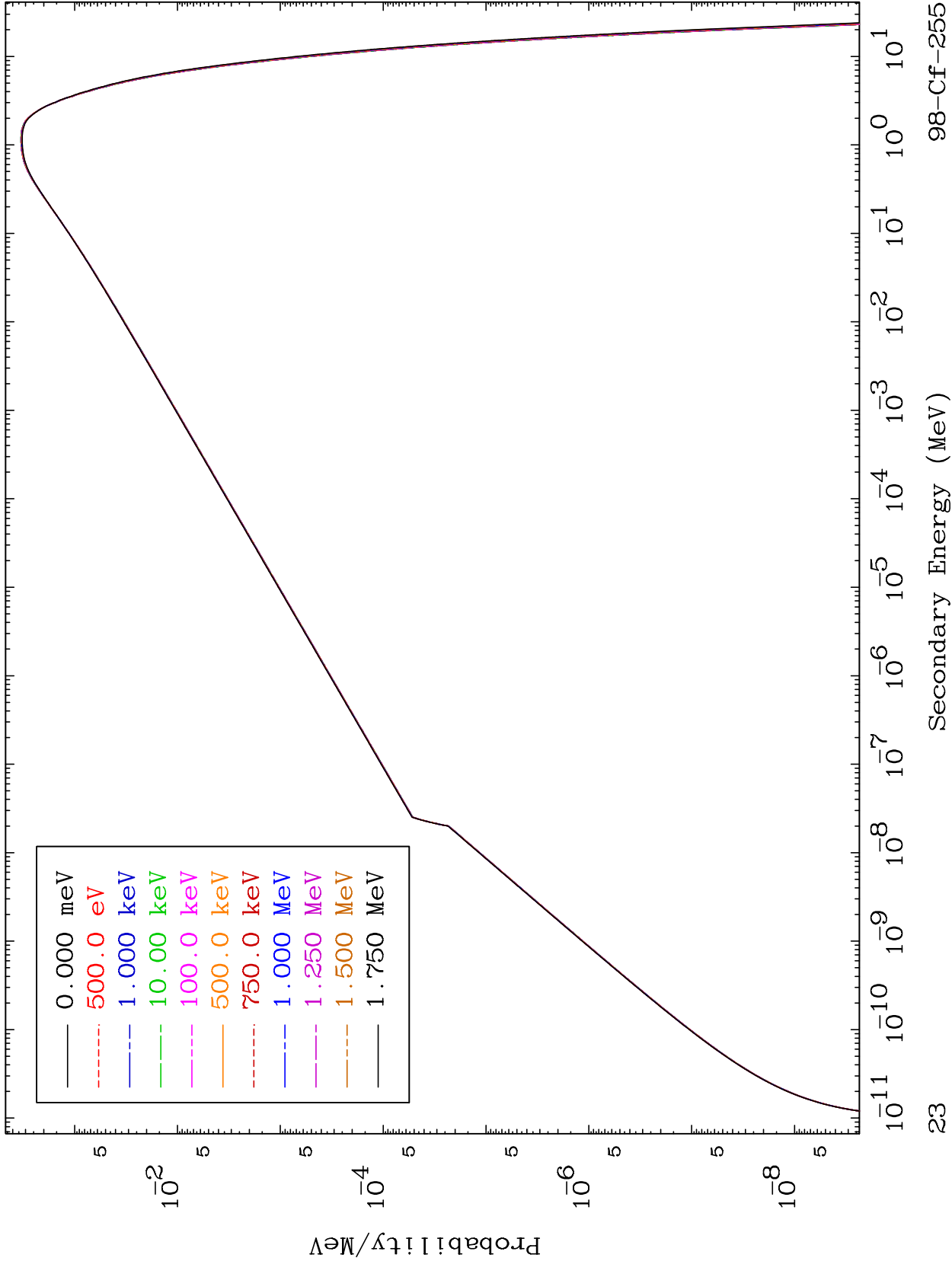
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Incident Energy (MeV)

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Fission Energy Distributions

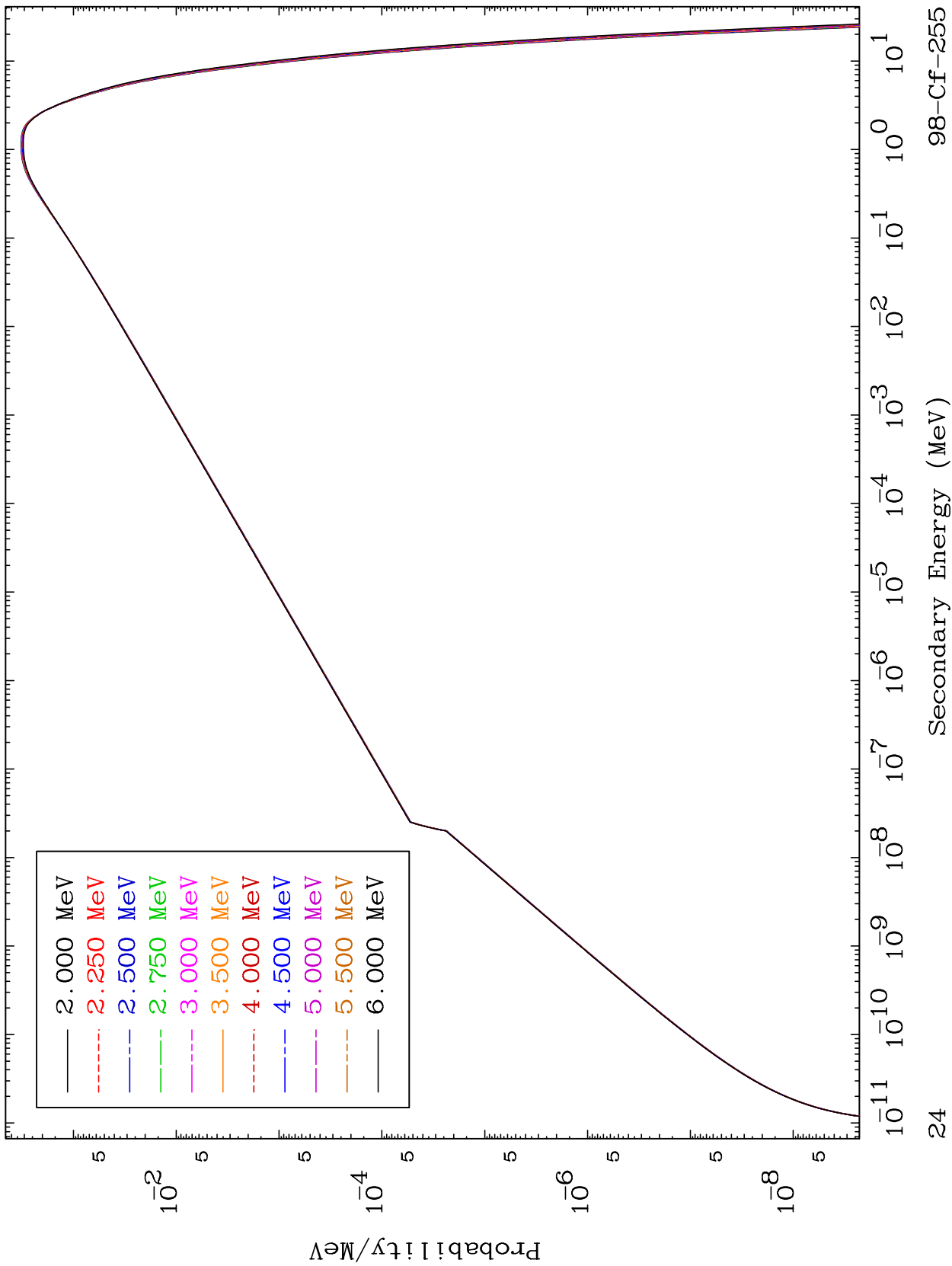
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Fission Energy Distributions

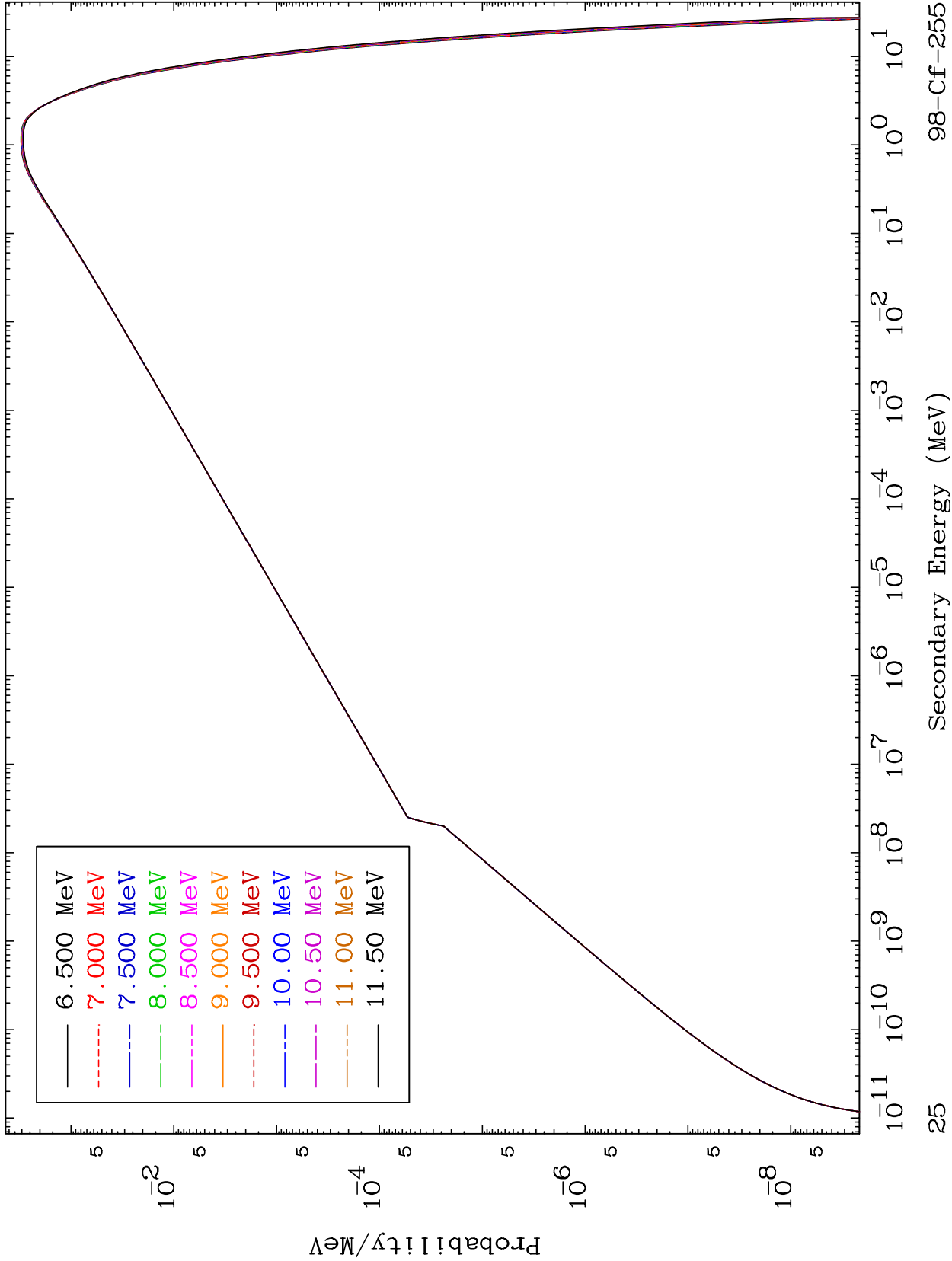
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Fission Energy Distributions

98-Cf-255



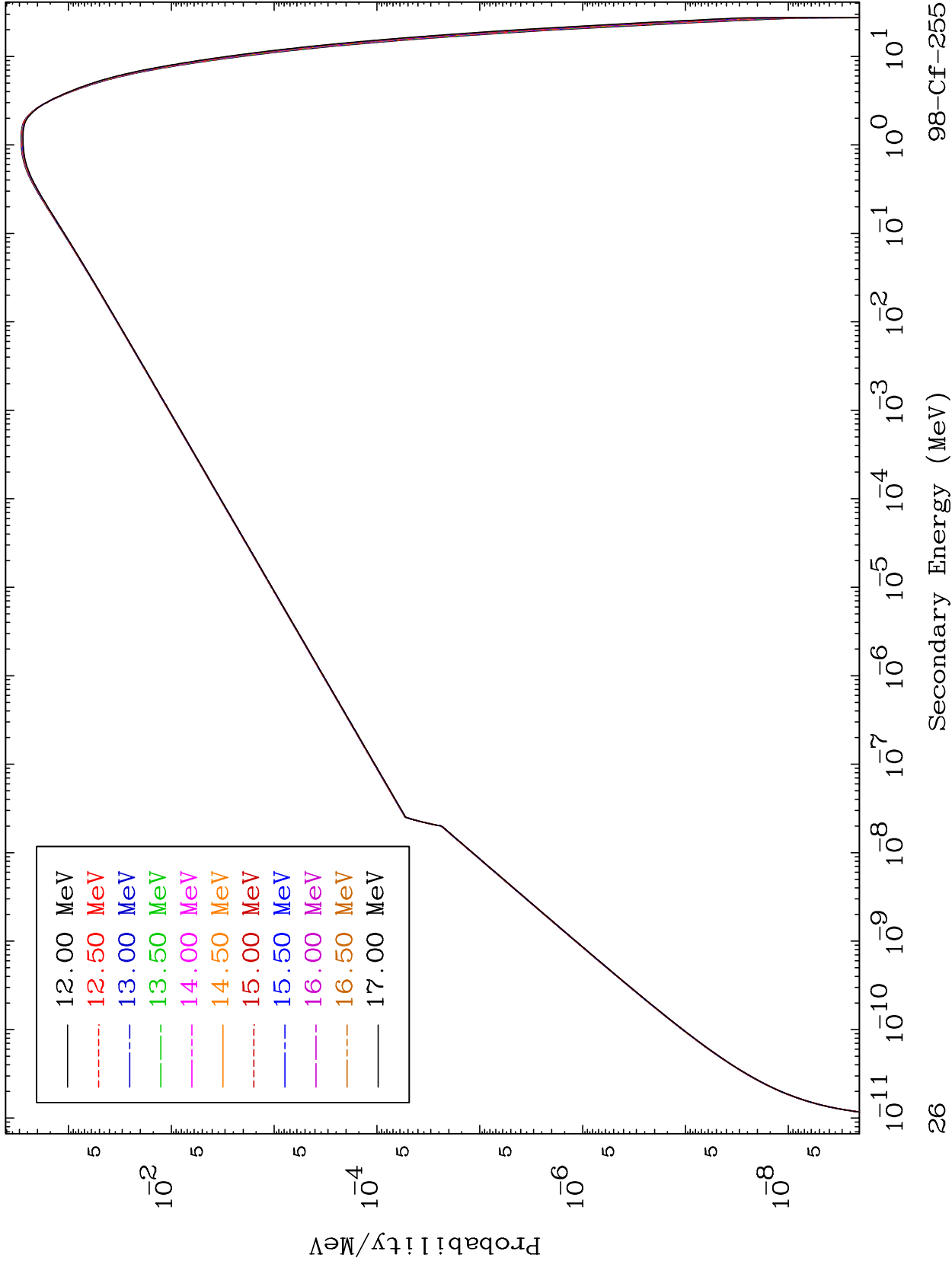
98-Cf-255

Secondary Energy (MeV)

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Fission Energy Distributions

98-Cf-255



98-Cf-255

Secondary Energy (MeV)

MAT 9870

Fission Energy Distributions

98-Cf-255

