

Program EVALPLOT
(Version 2021-1)

by

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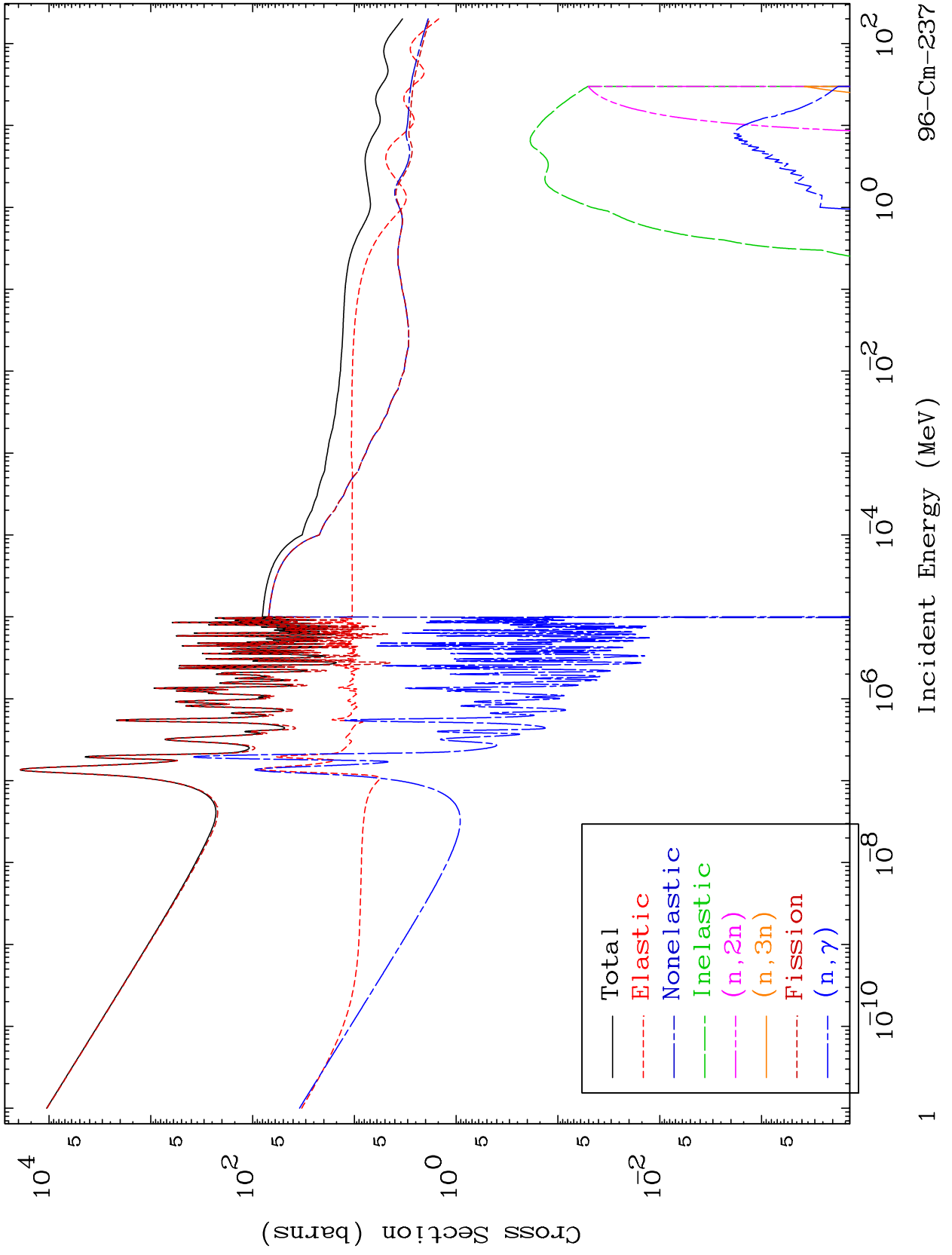
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Press Mouse Button to Start

MAT 9616

Neutron Major
293 Kelvin Cross Sections

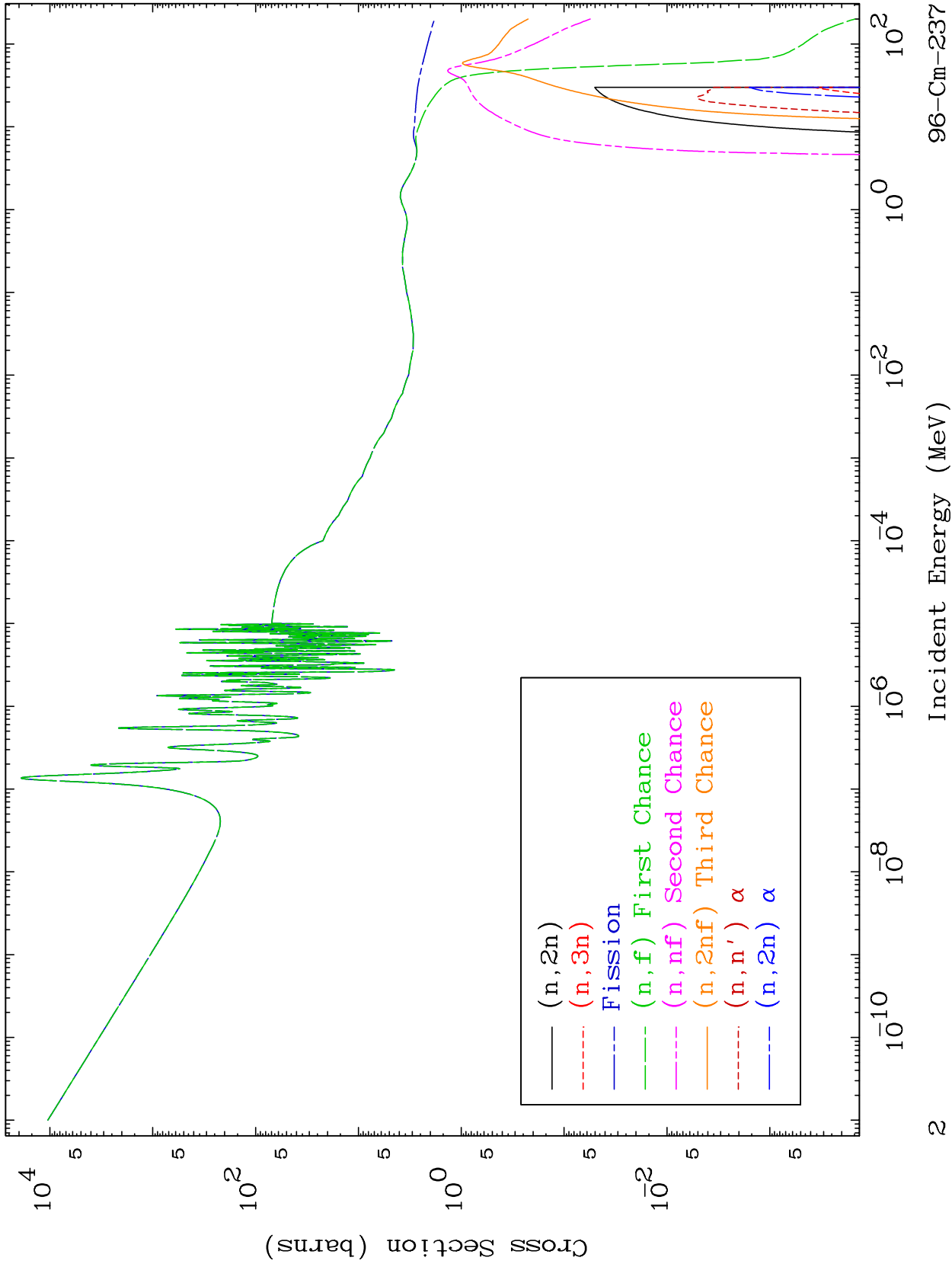
96-Cm-237



MAT 9616

Neutron Absorption
293 Kelvin Cross Sections

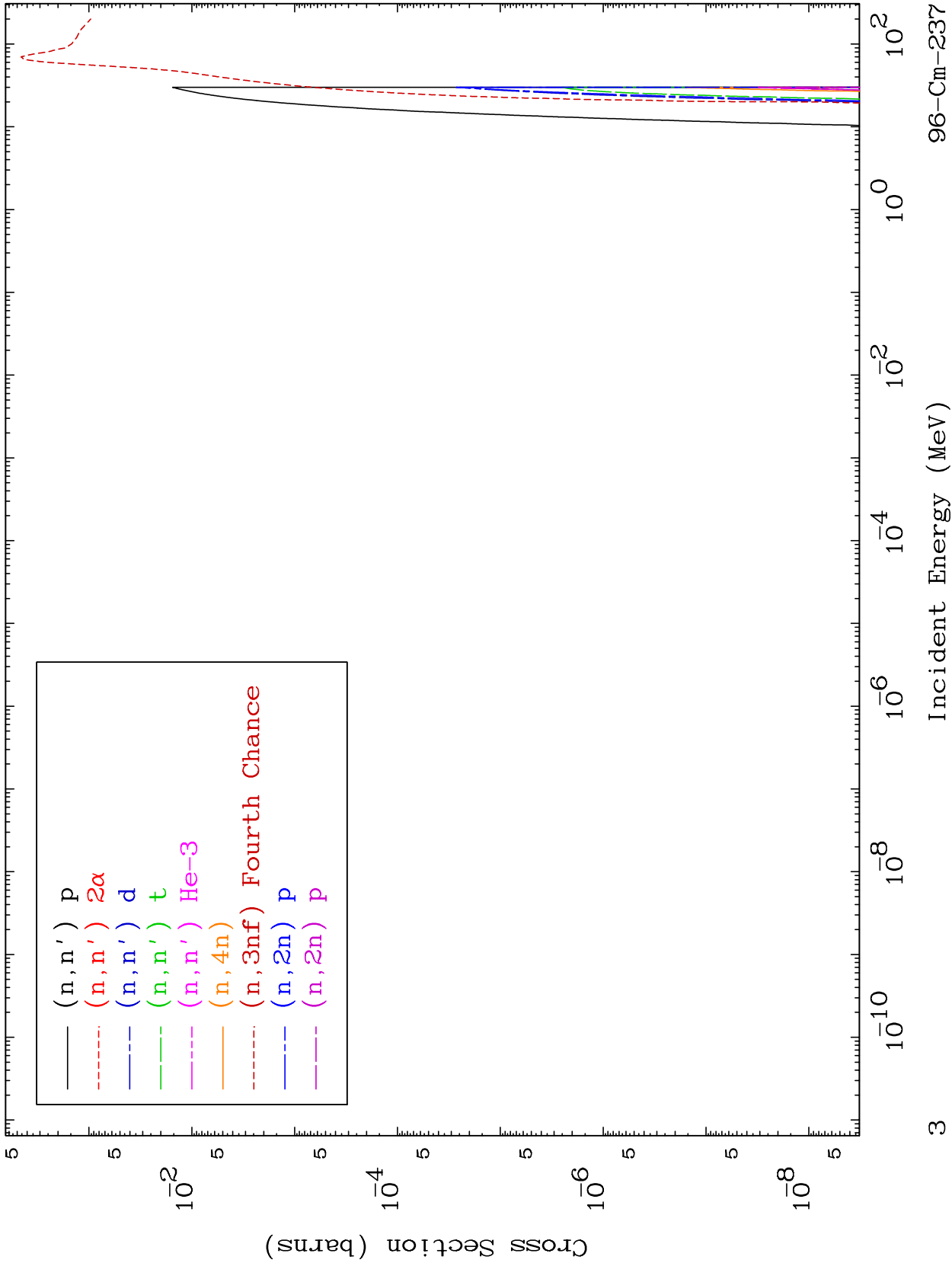
96-Cm-237



MAT 9616

Neutron Absorption
293 Kelvin Cross Sections

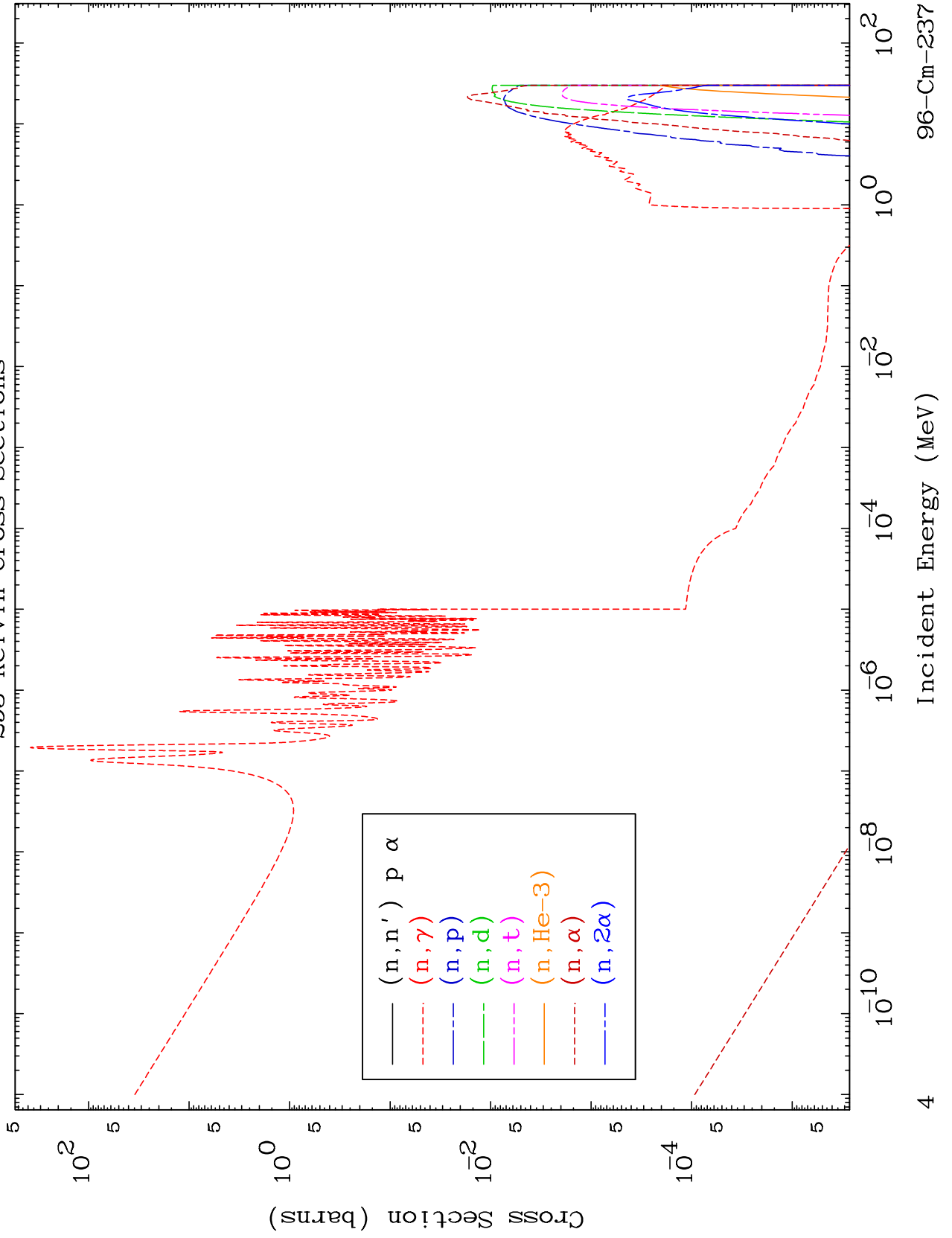
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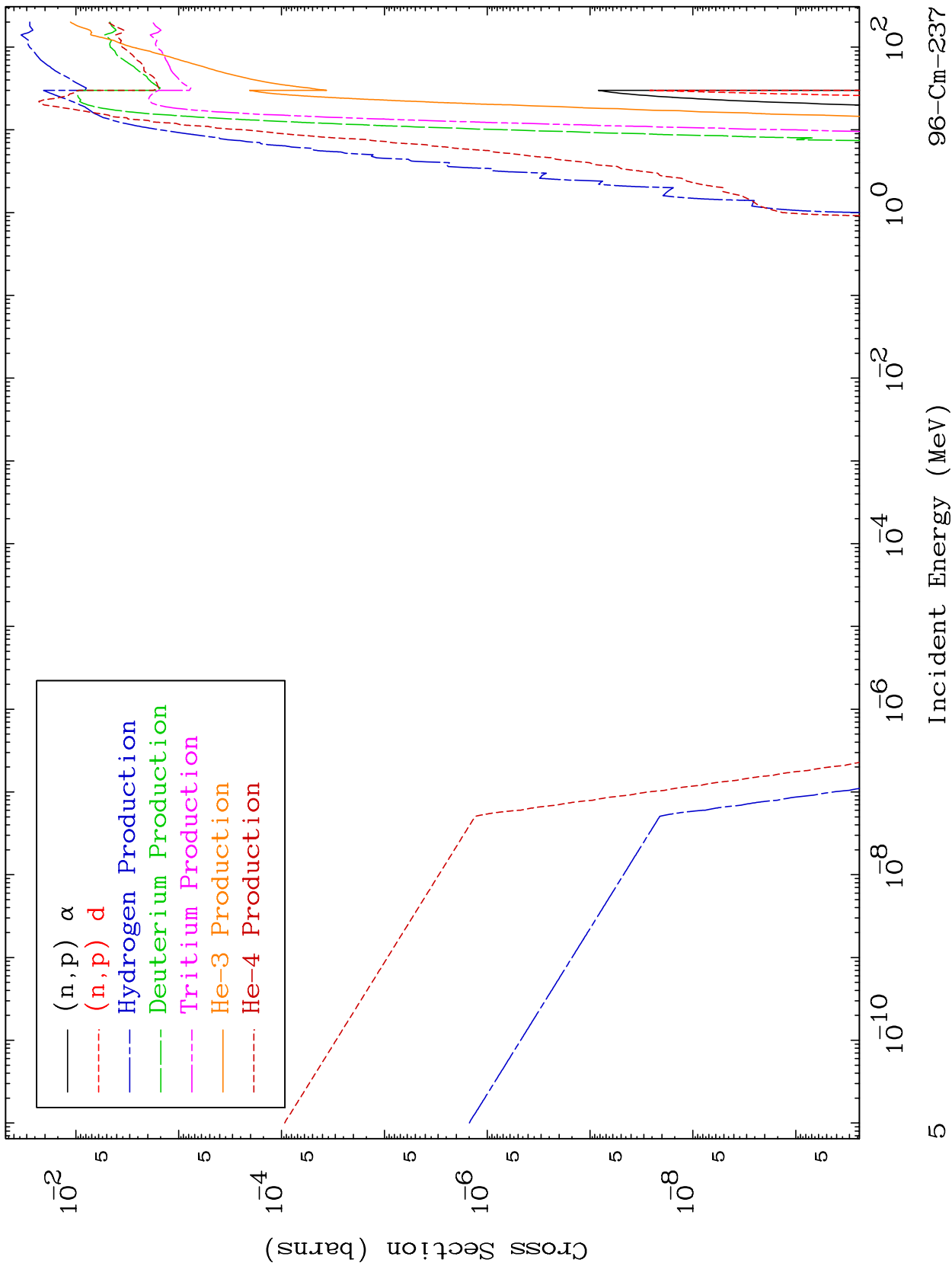


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Neutron Absorption
293 Kelvin Cross Sections

96-Cm-237

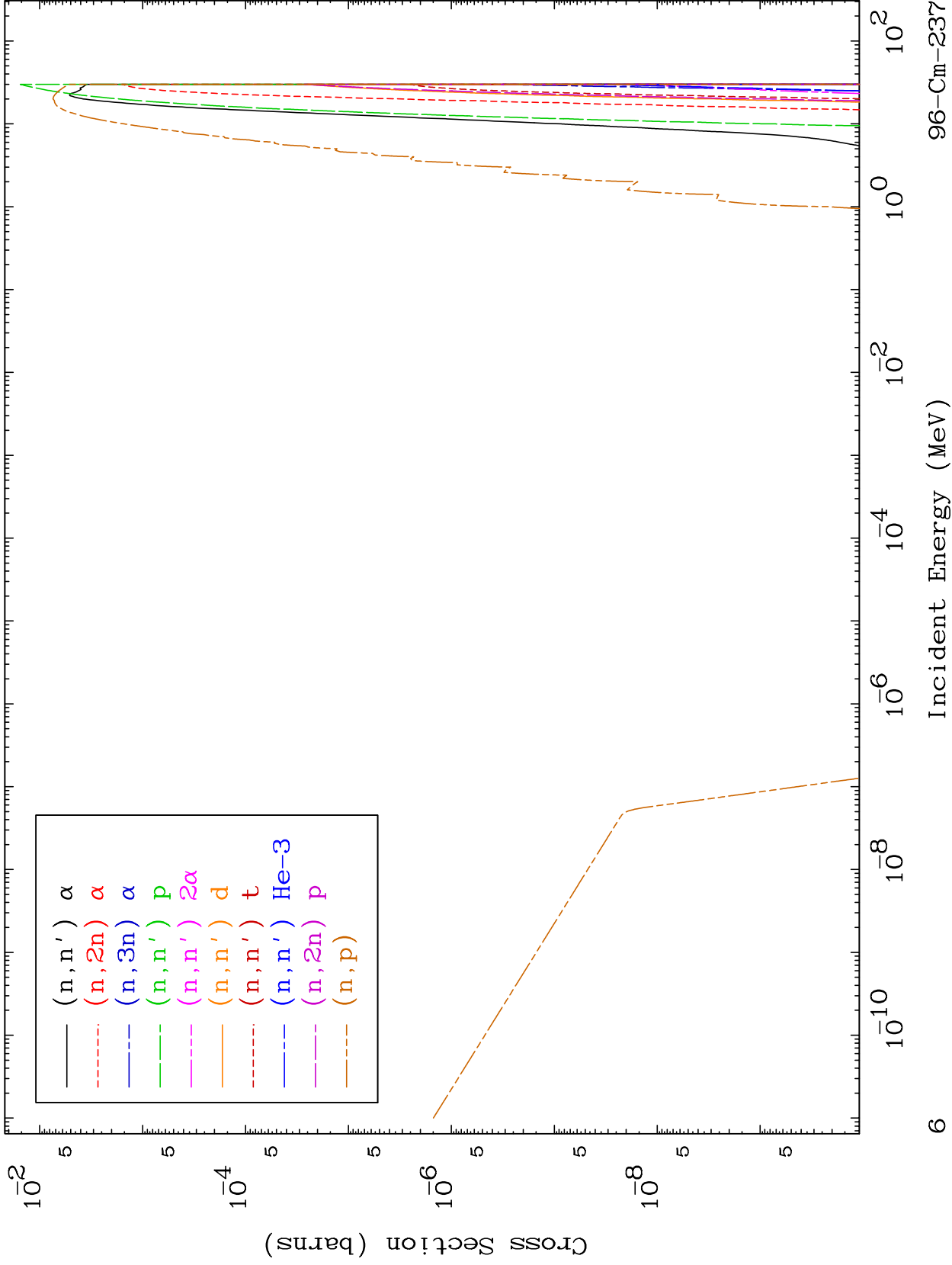




MAT 9616

Charged Particle
293 Kelvin Cross Sections

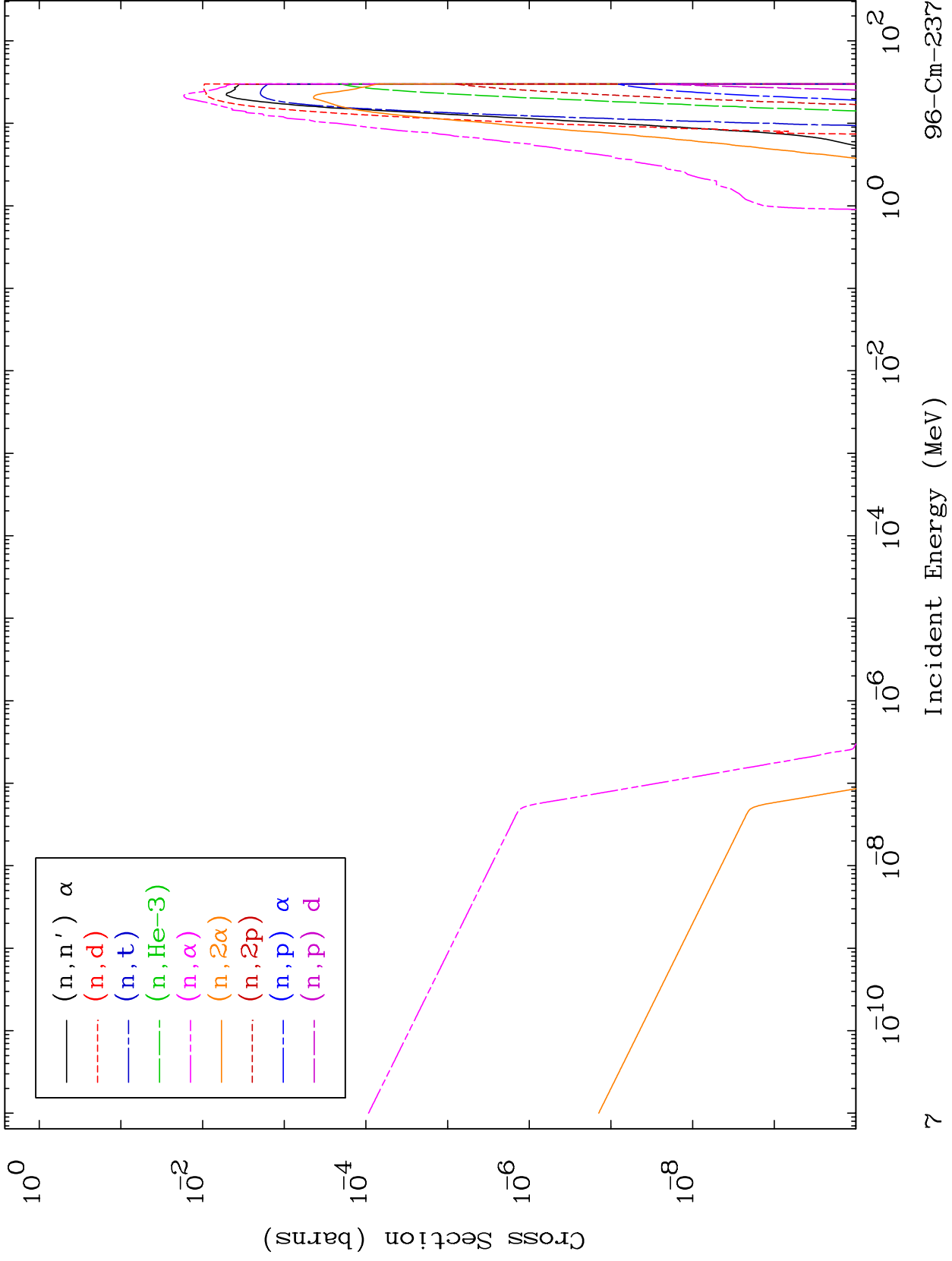
96-Cm-237



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Charged Particle
293 Kelvin Cross Sections

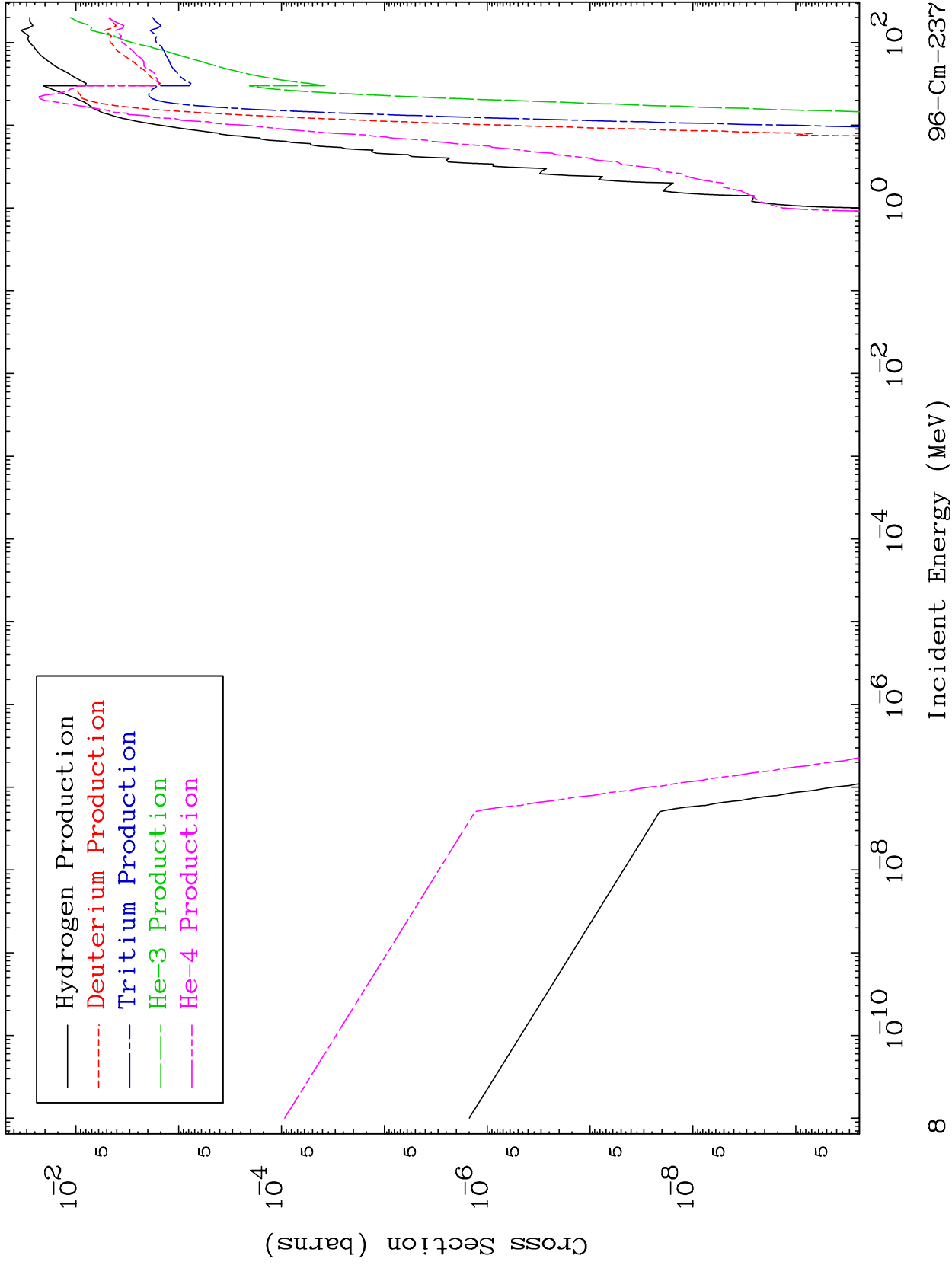
96-Cm-237



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Particle Production
293 Kelvin Cross Sections

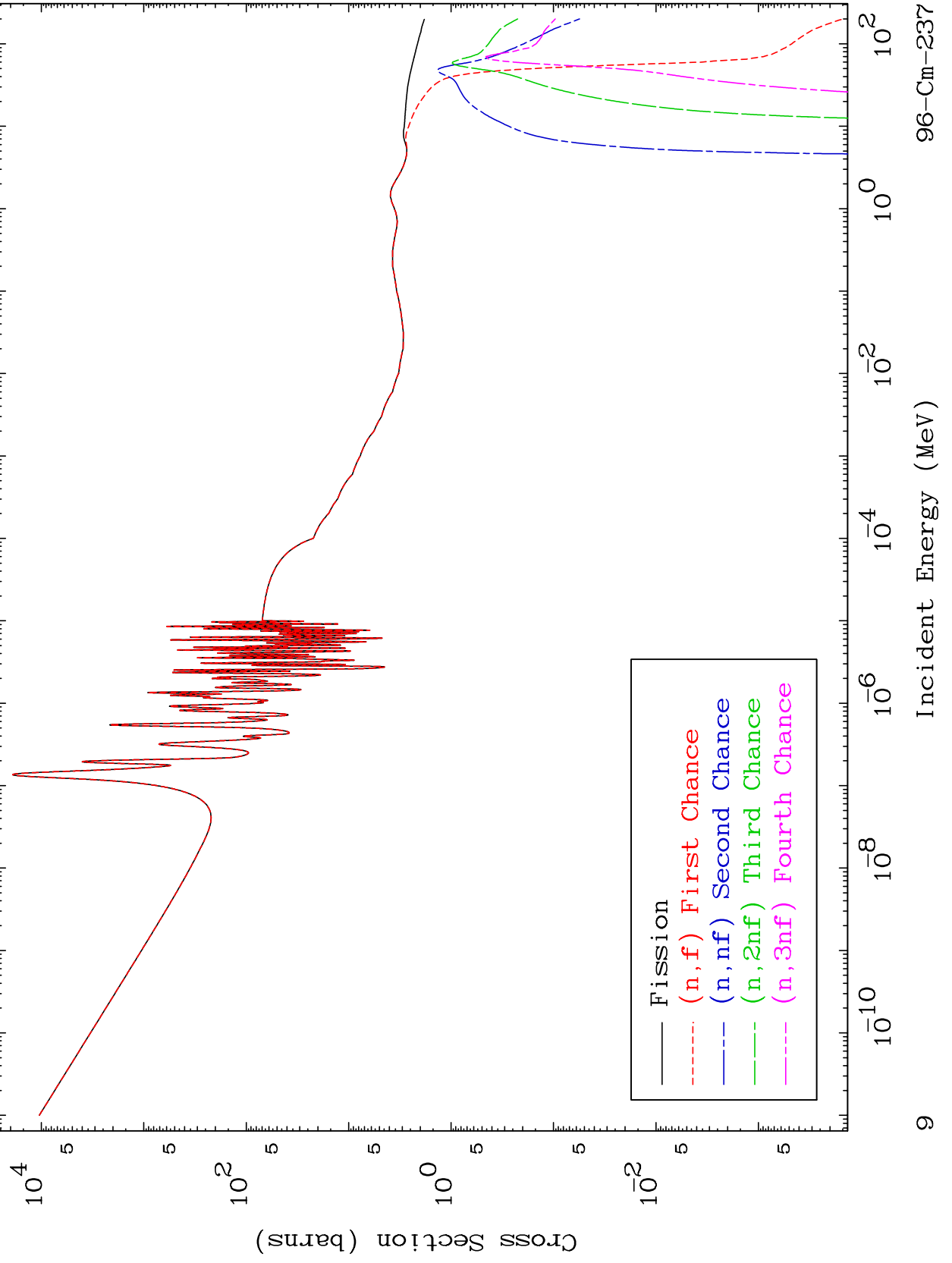
96-Cm-237



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Fission
293 Kelvin Cross Sections

96-Cm-237

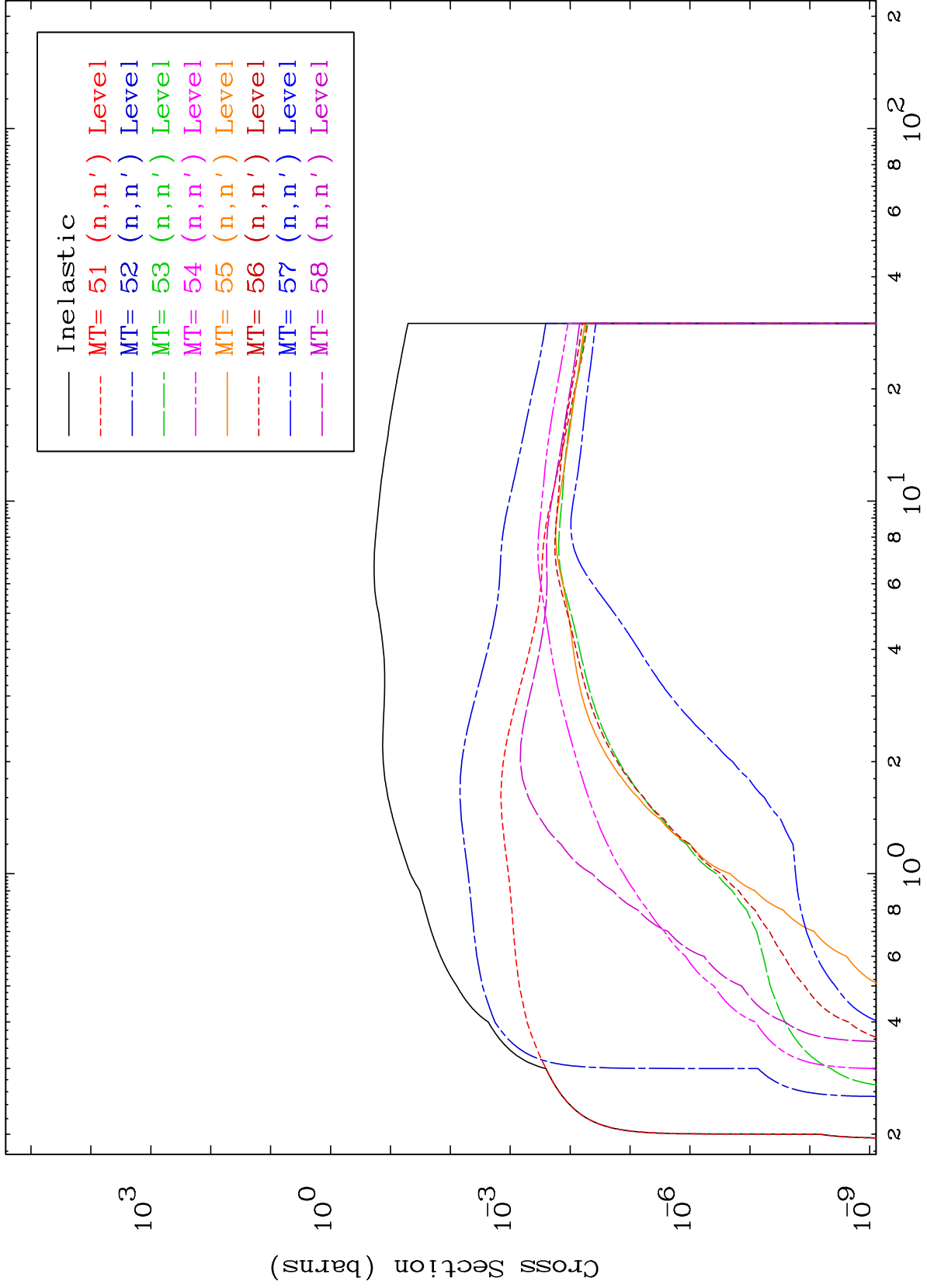


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(n,n') Levels

96-Cm-237

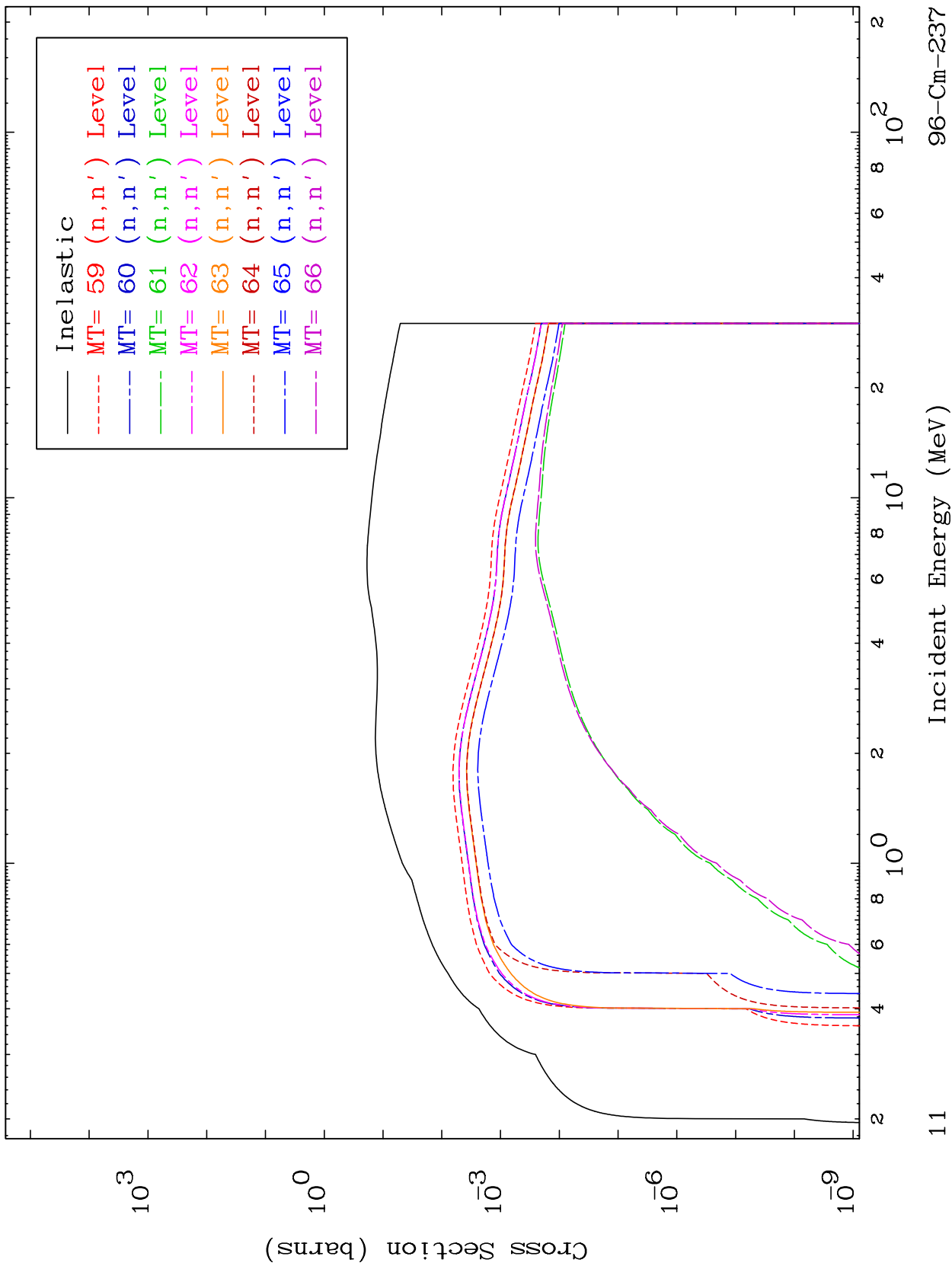
293 Kelvin Cross Sections



10

Incident Energy (MeV)

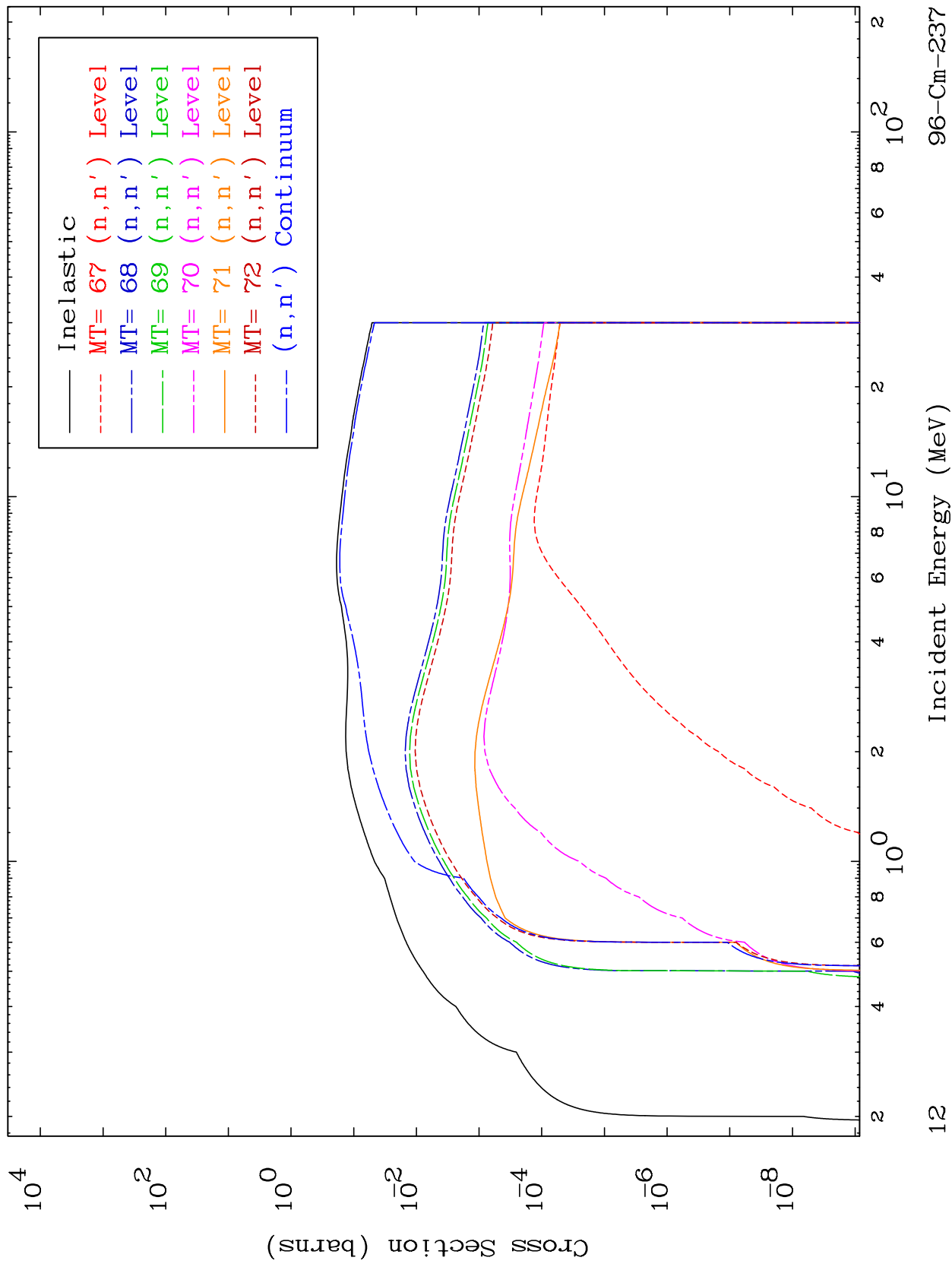
96-Cm-237



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96-Cm-237

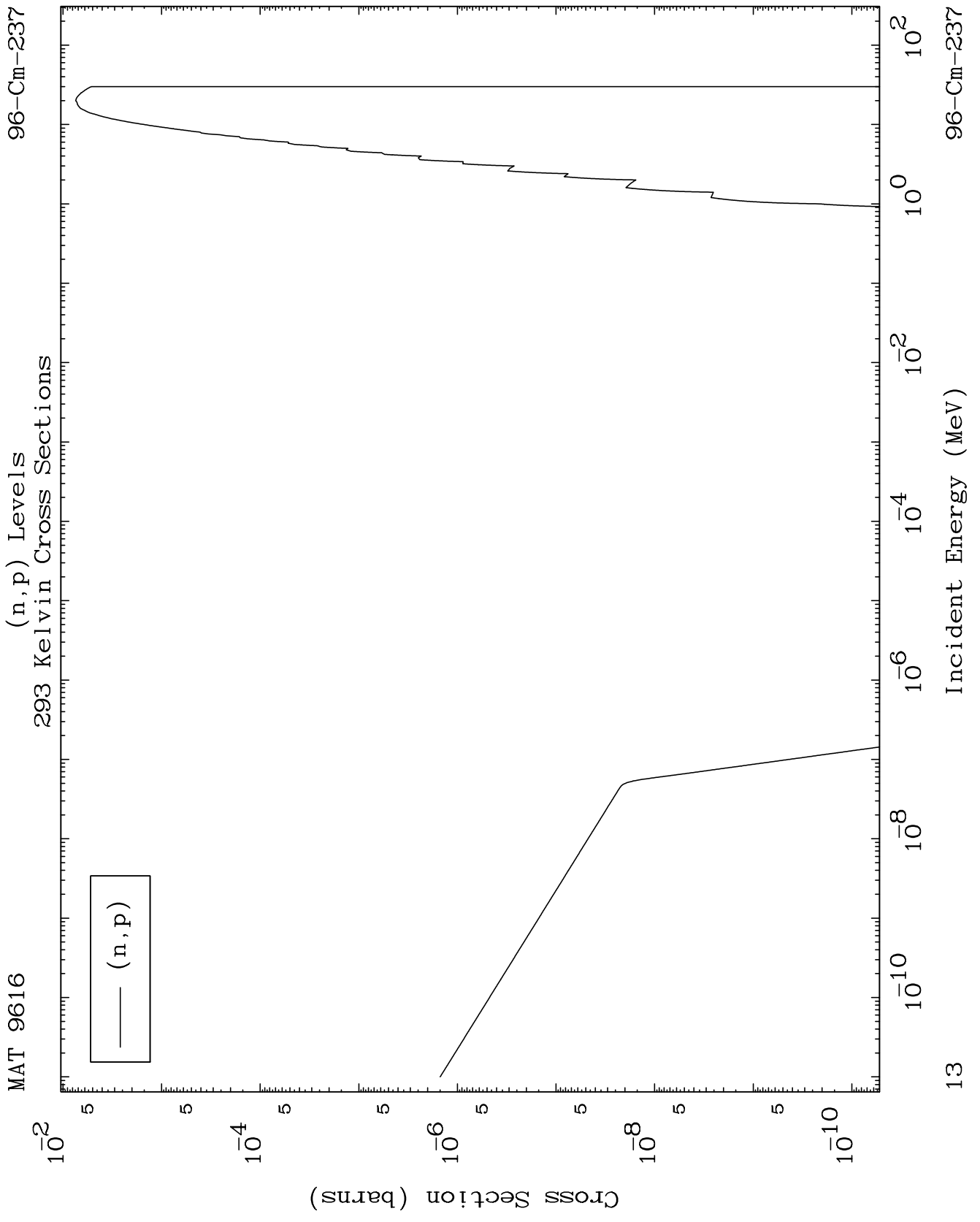
(n,n') Levels
293 Kelvin Cross Sections



96-Cm-237

Incident Energy (MeV)

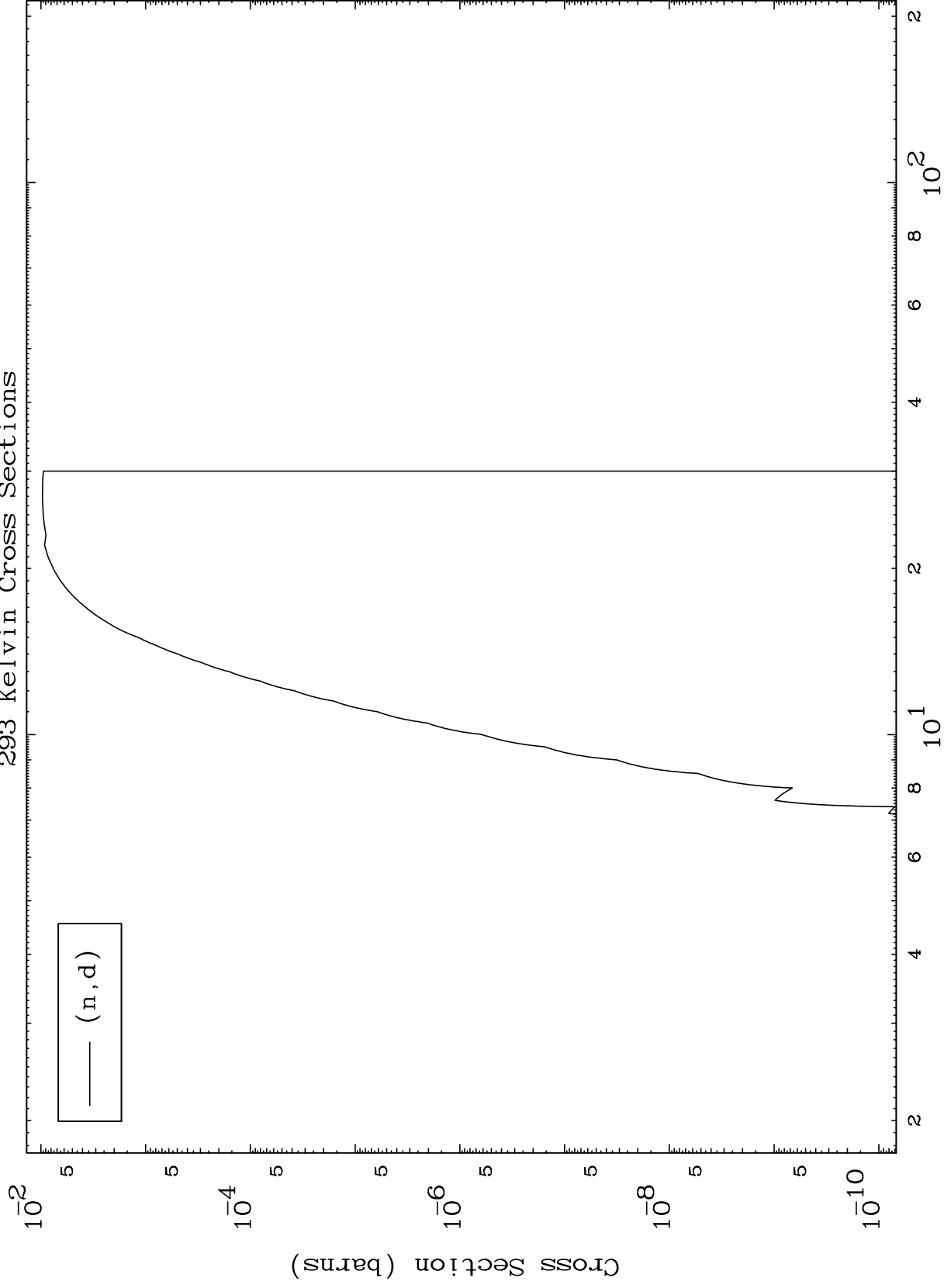
12



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(n,d) Levels
293 Kelvin Cross Sections

96-Cm-237



14

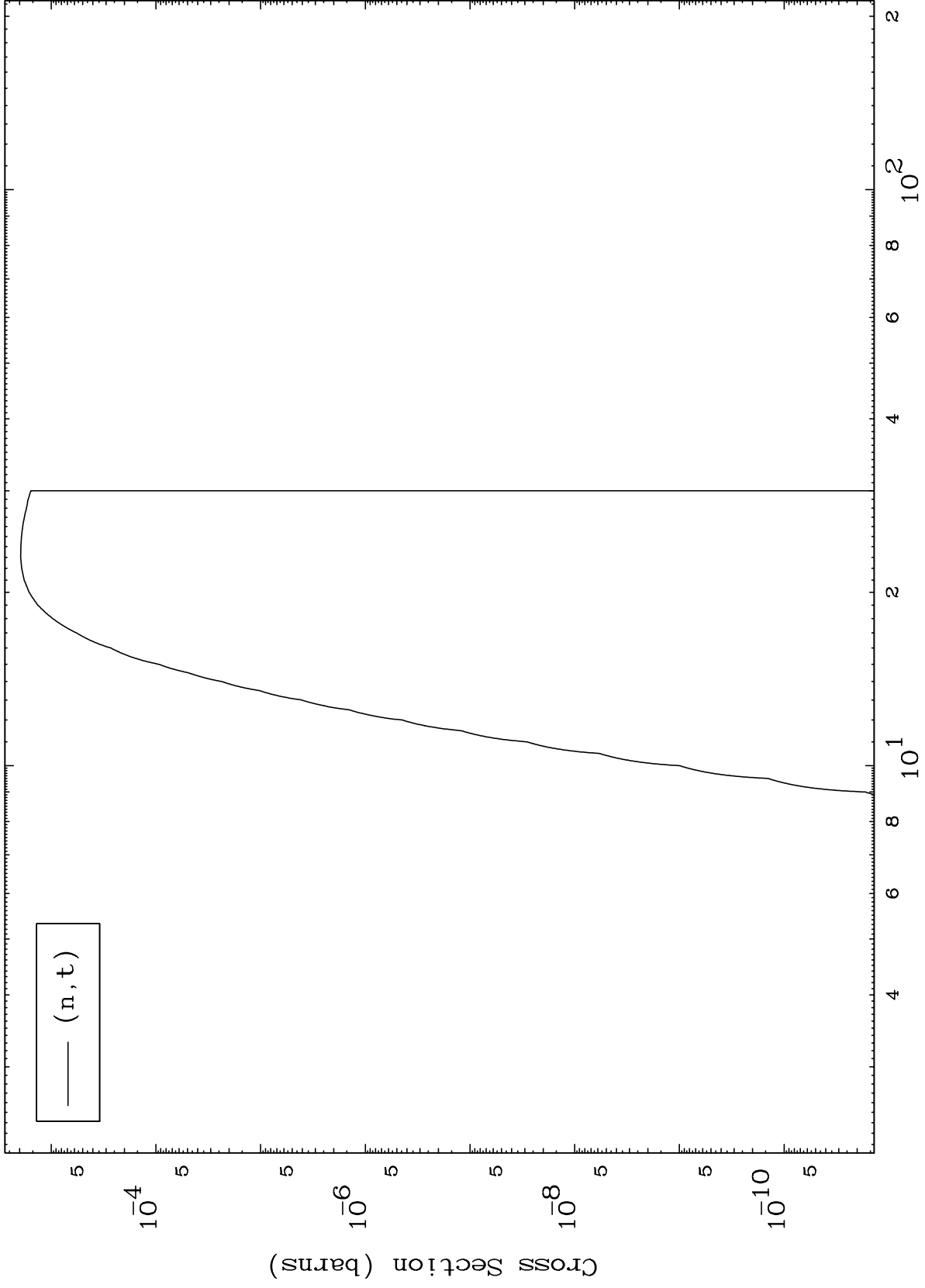
Incident Energy (MeV)

96-Cm-237

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(n,t) Levels
293 Kelvin Cross Sections

96-Cm-237



15

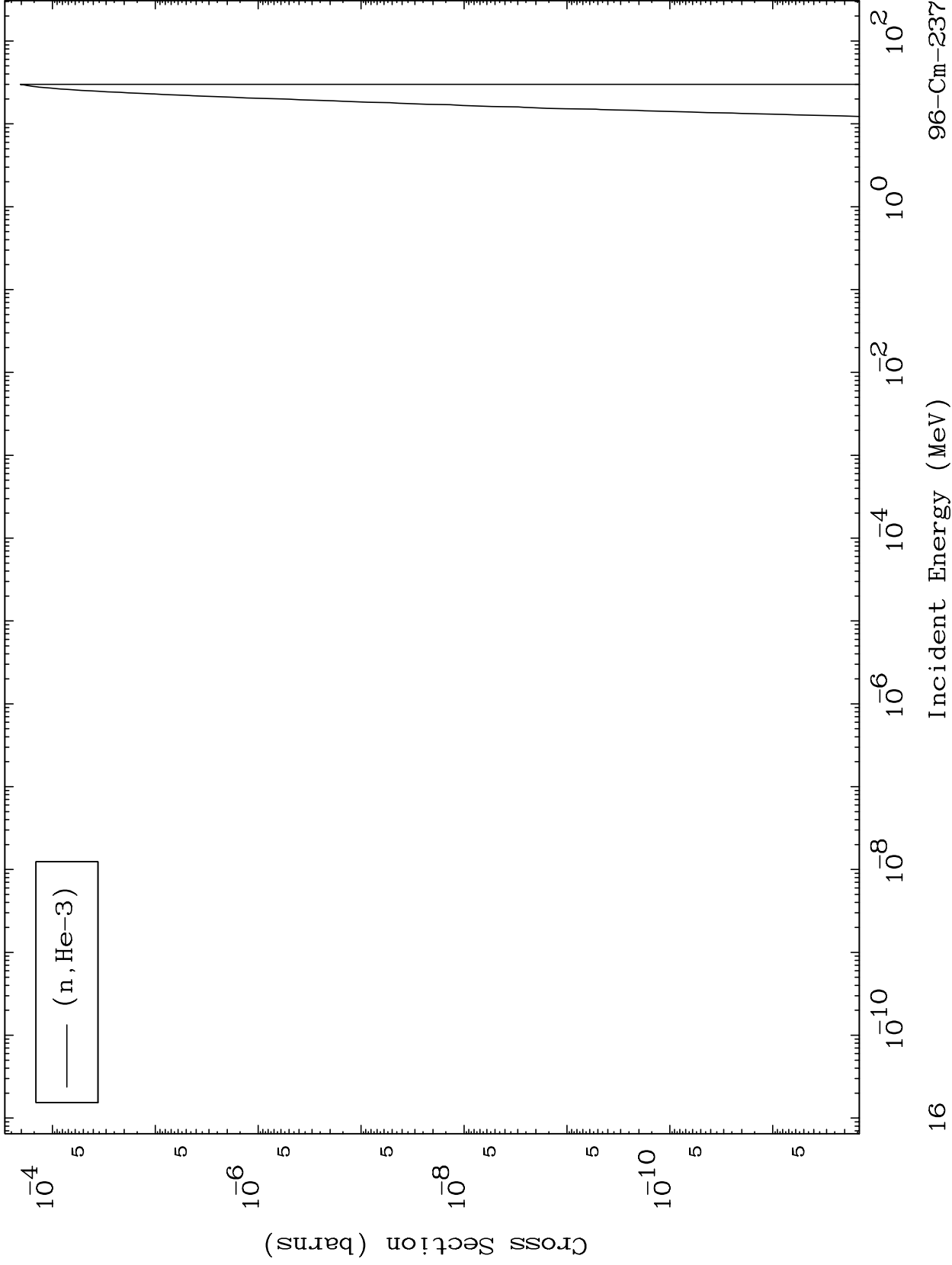
Incident Energy (MeV)

96-Cm-237

MAT 9616

(n,He3) Levels
293 Kelvin Cross Sections

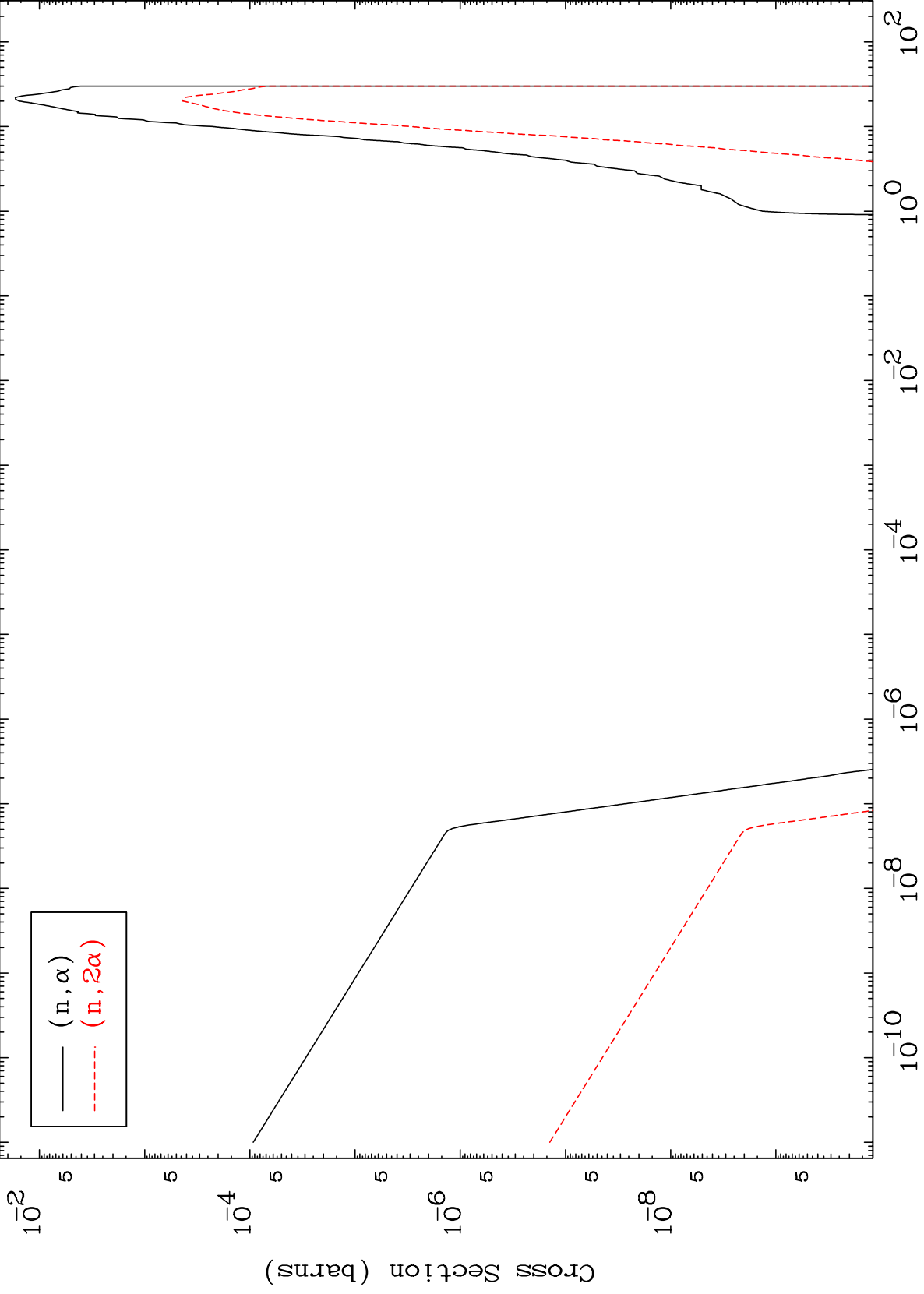
96-Cm-237



MAT 9616

(n, α) Levels
293 Kelvin Cross Sections

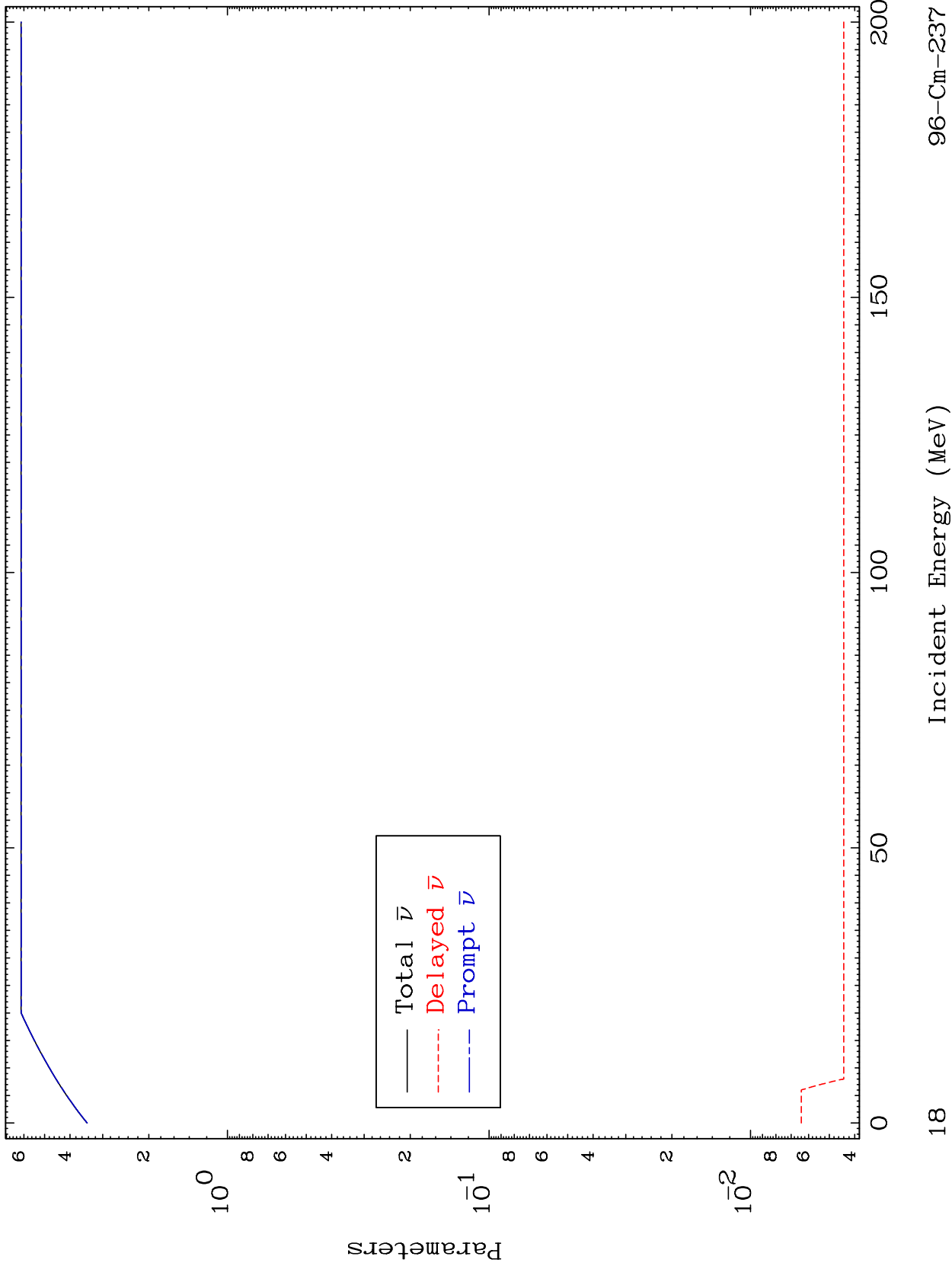
96-Cm-237



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Energy Release
Parameters

96-Cm-237



18

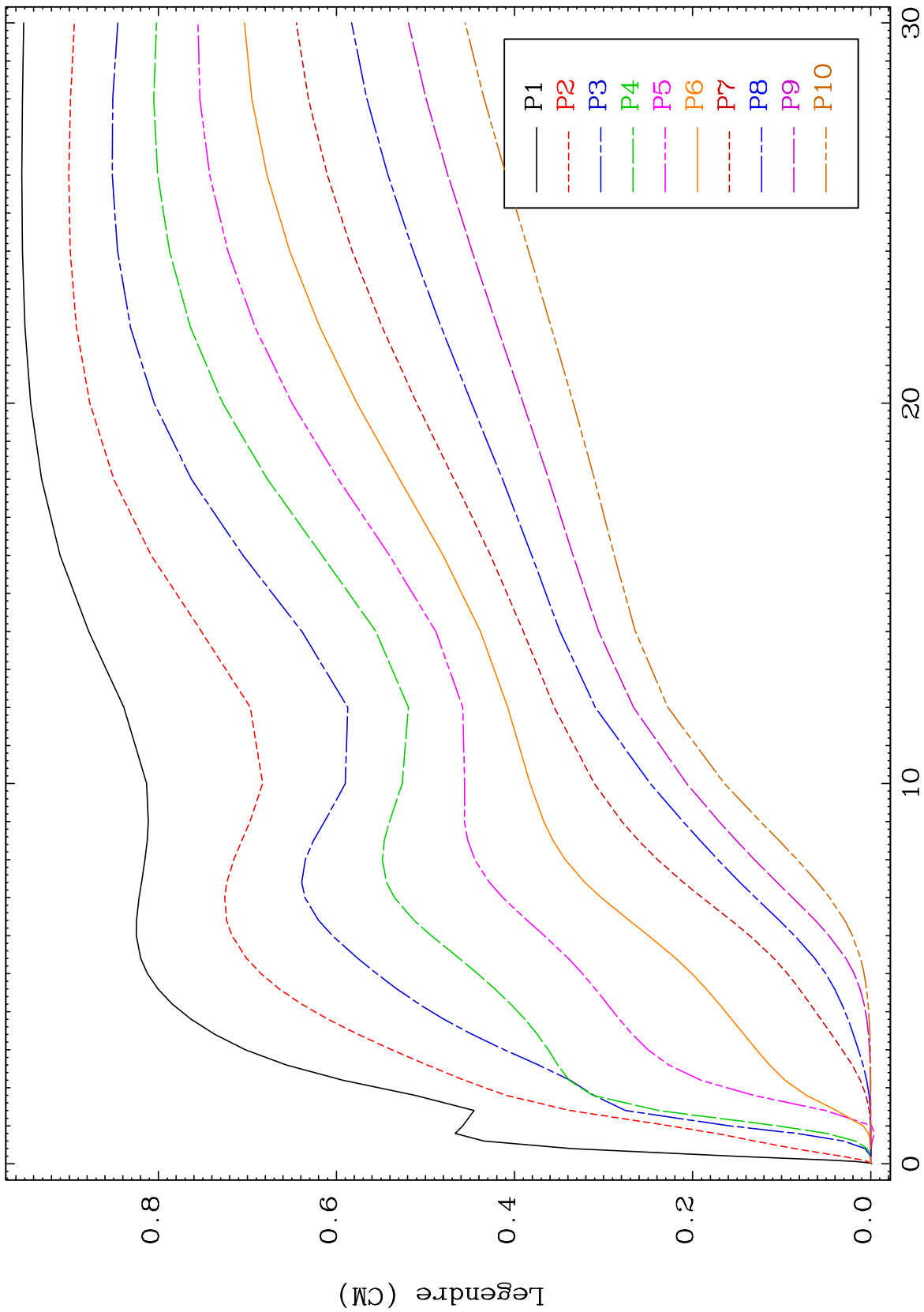
Incident Energy (MeV)

96-Cm-237

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96-Cm-237

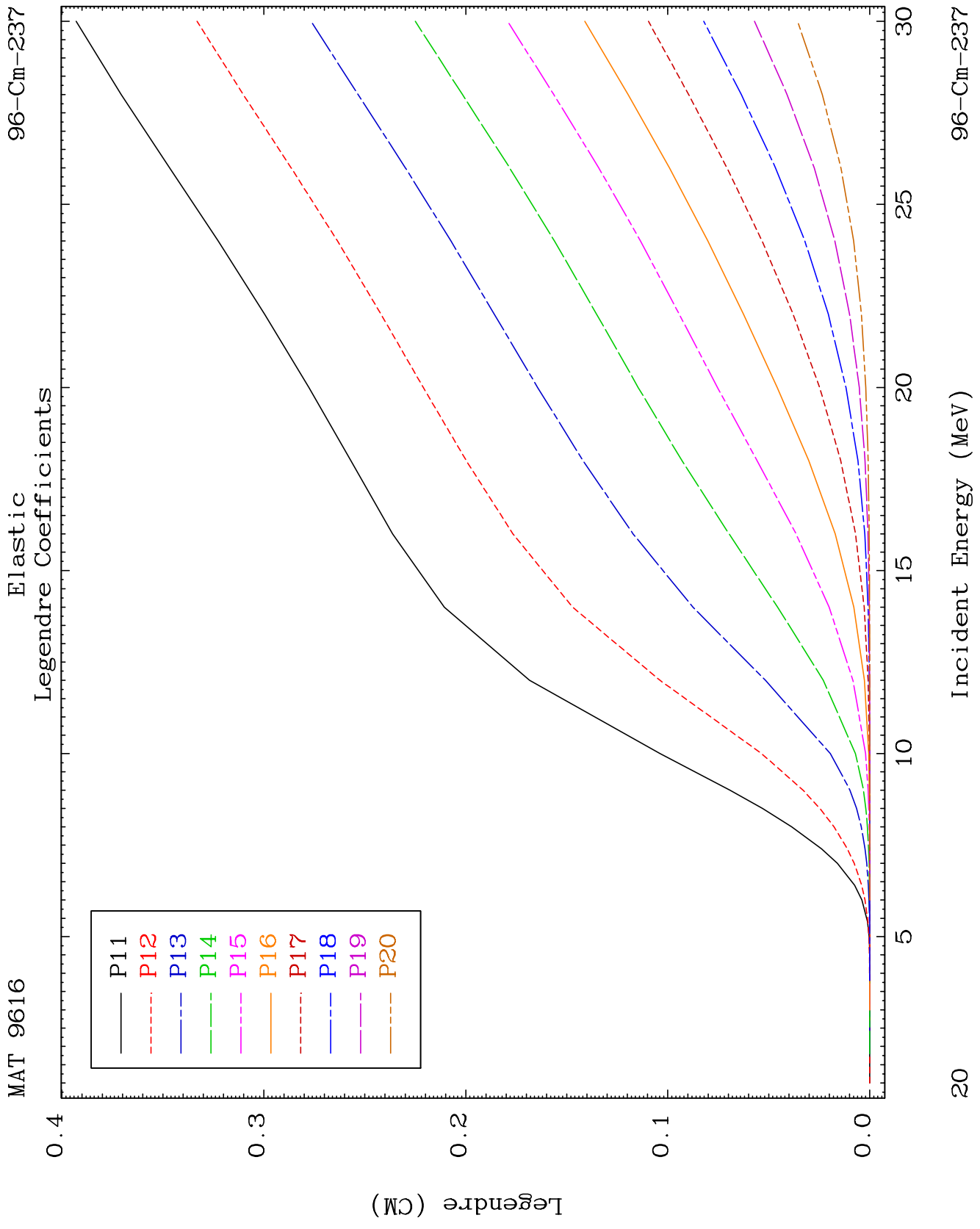
Elastic Legendre Coefficients



19

Incident Energy (MeV)

96-Cm-237



20

5

10

15

20

25

30

0.4

0.3

0.2

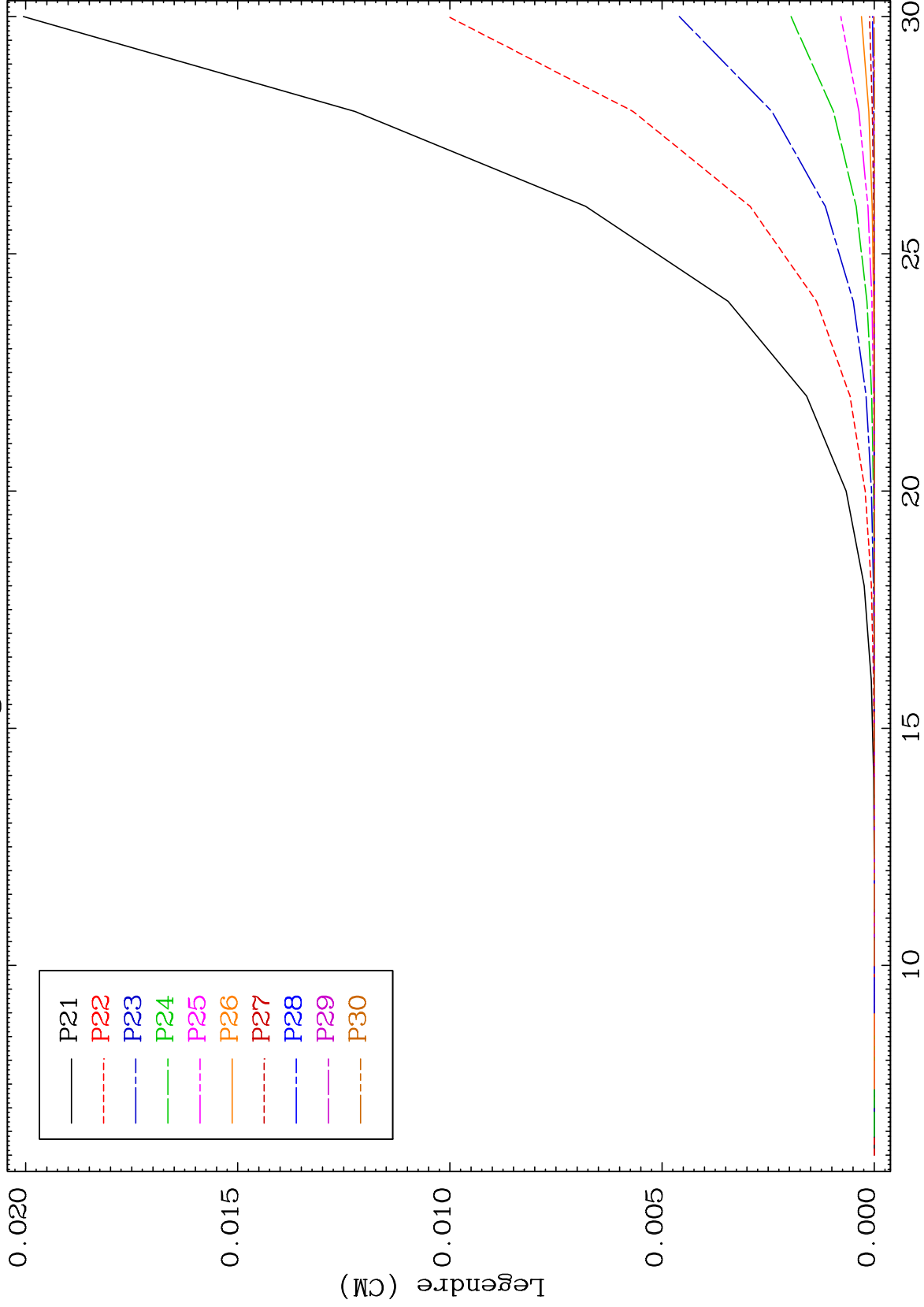
0.1

0.0

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Elastic
Legendre Coefficients

96-Cm-237



21

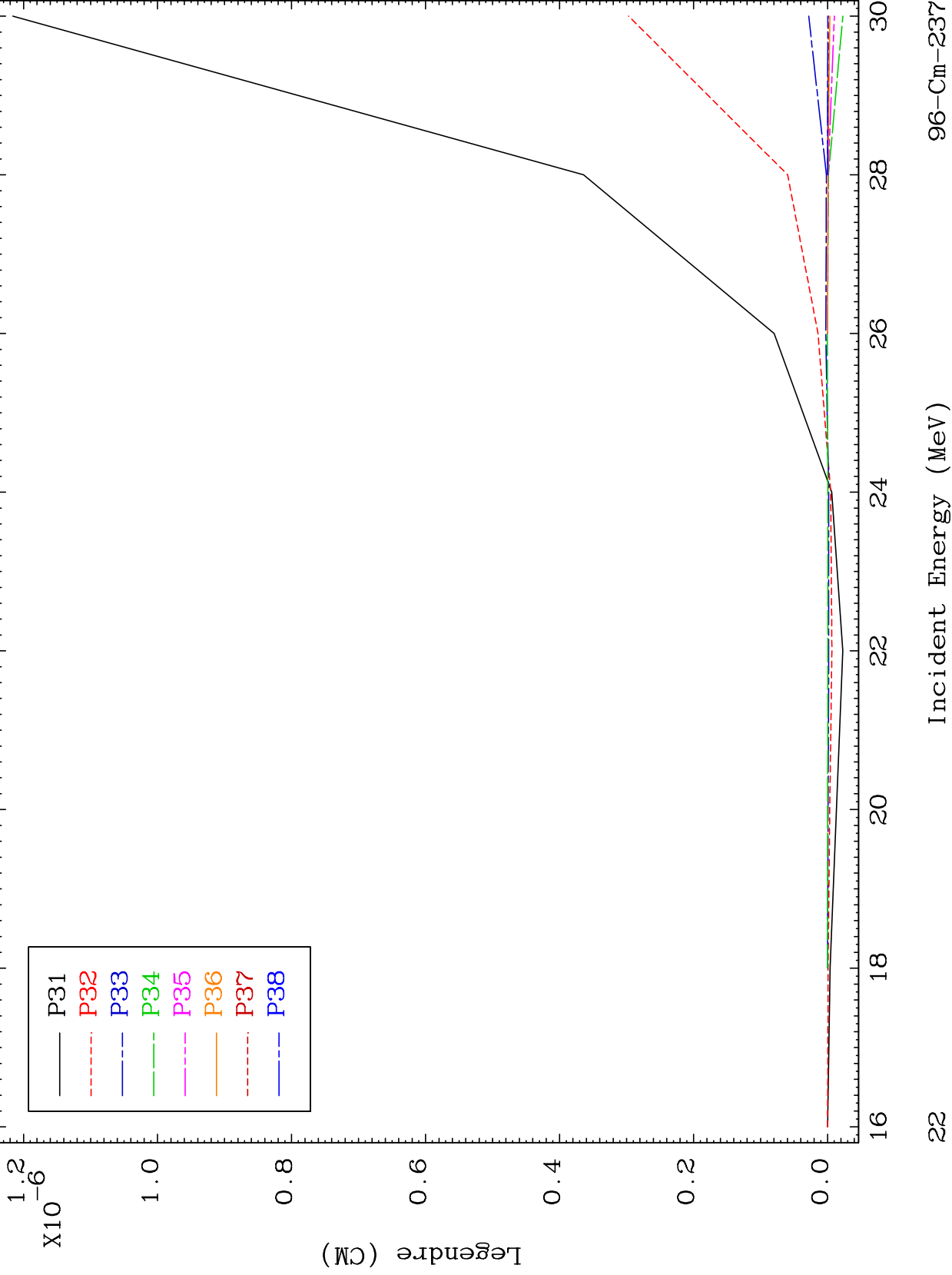
Incident Energy (MeV)

96-Cm-237

MAT 9616

Elastic
Legendre Coefficients

96-Cm-237

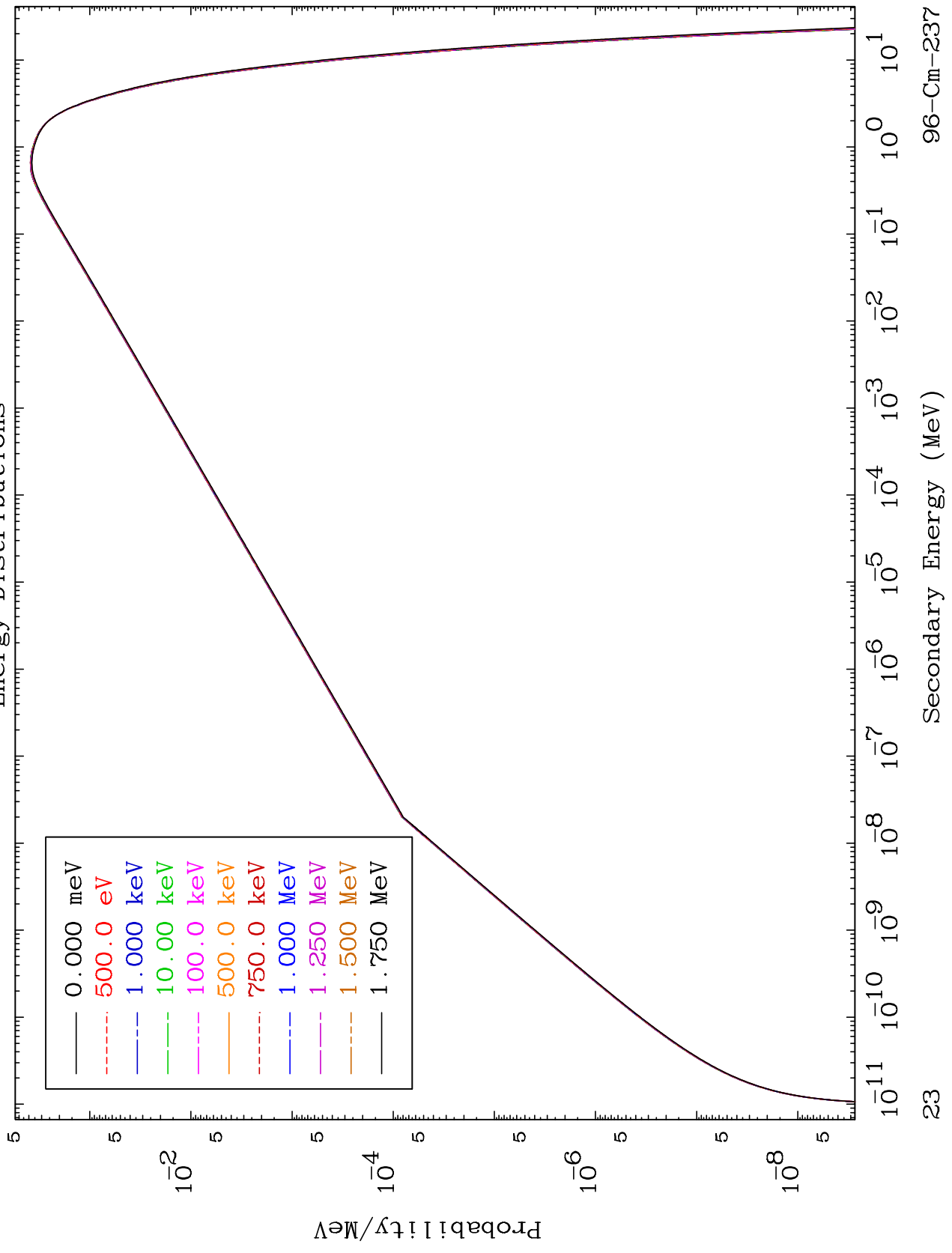


96-Cm-237

Incident Energy (MeV)

22

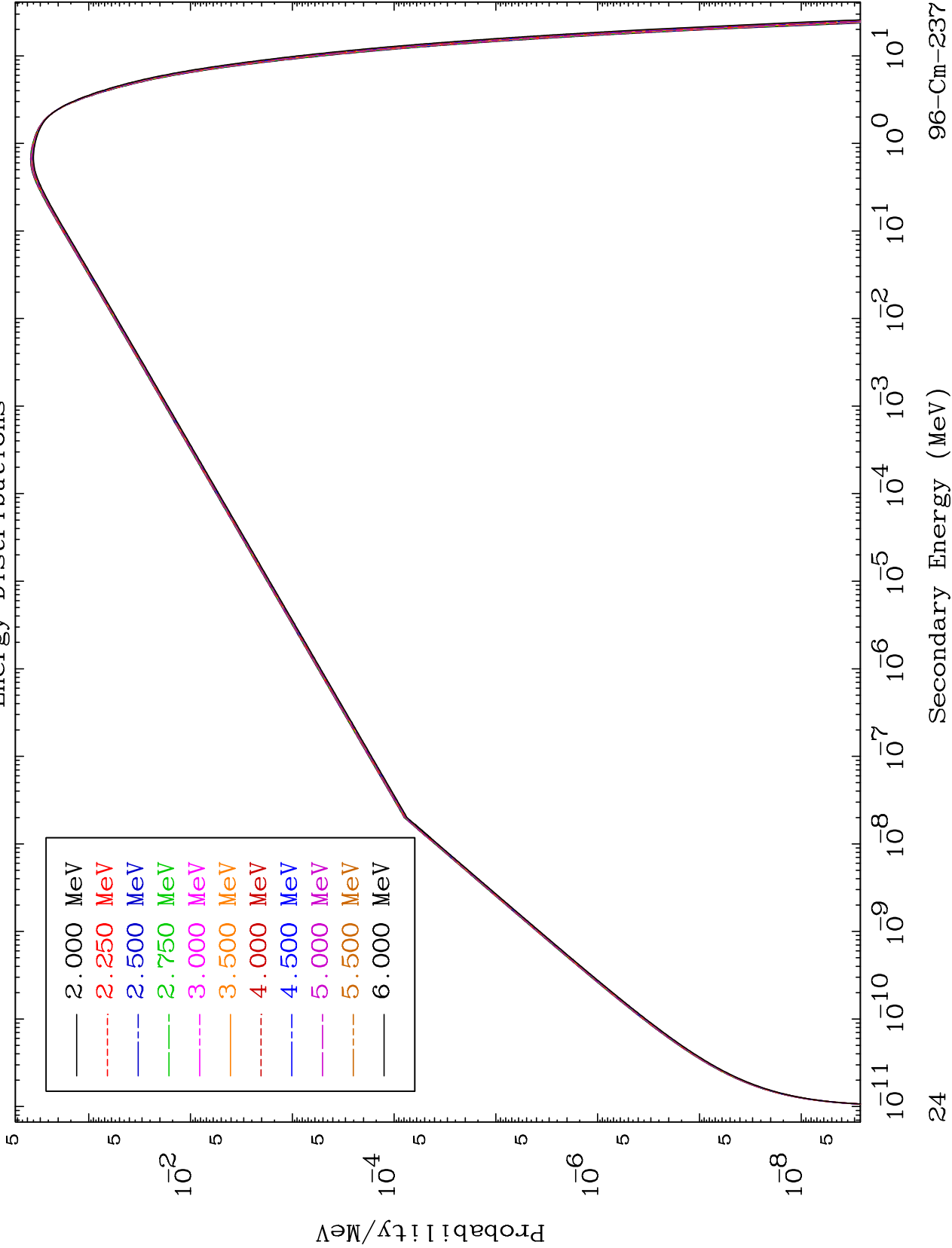
Fission Energy Distributions



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Fission Energy Distributions

96-Cm-237



96-Cm-237

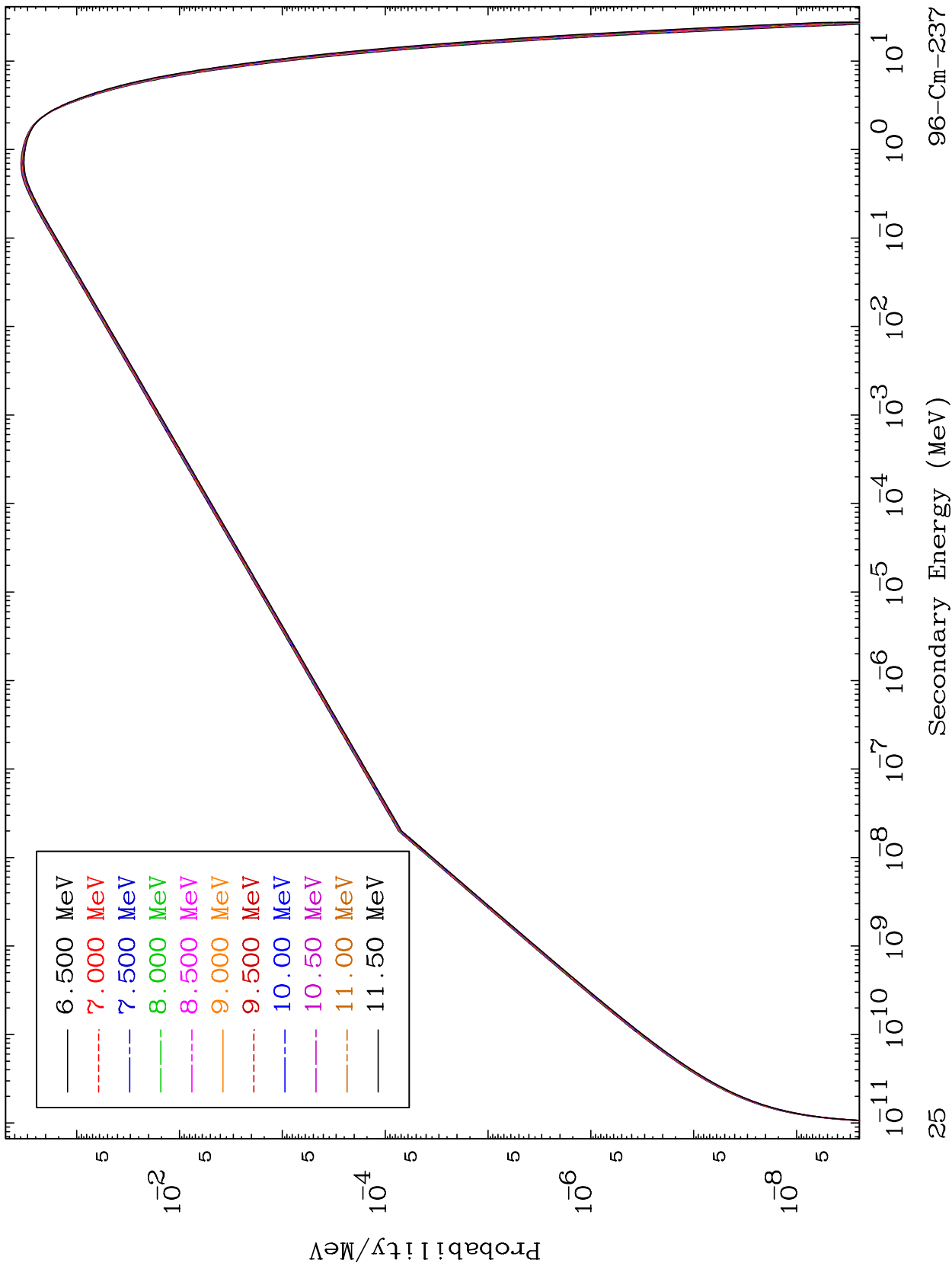
Secondary Energy (MeV)

24

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Fission Energy Distributions

96-Cm-237

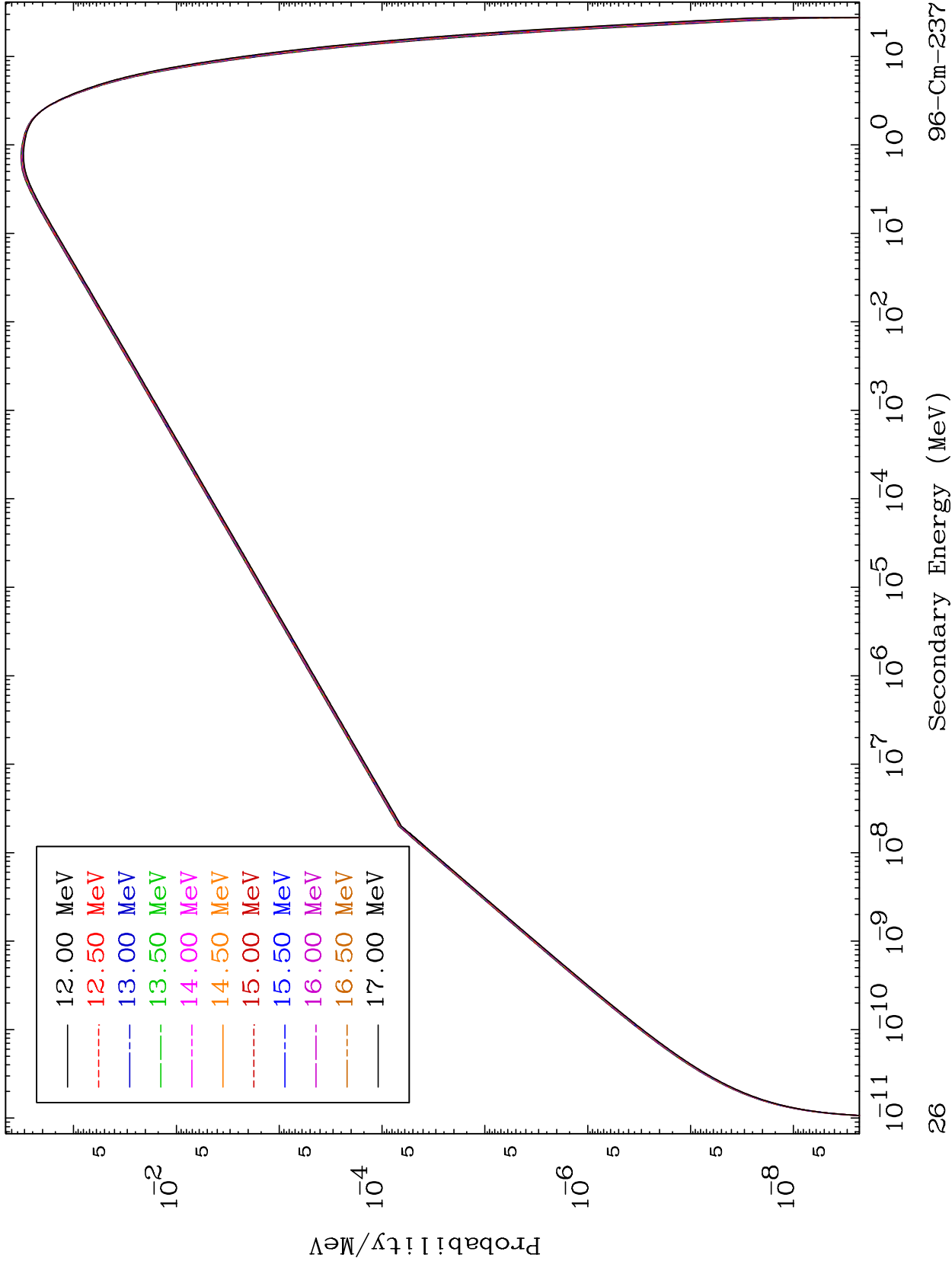


96-Cm-237

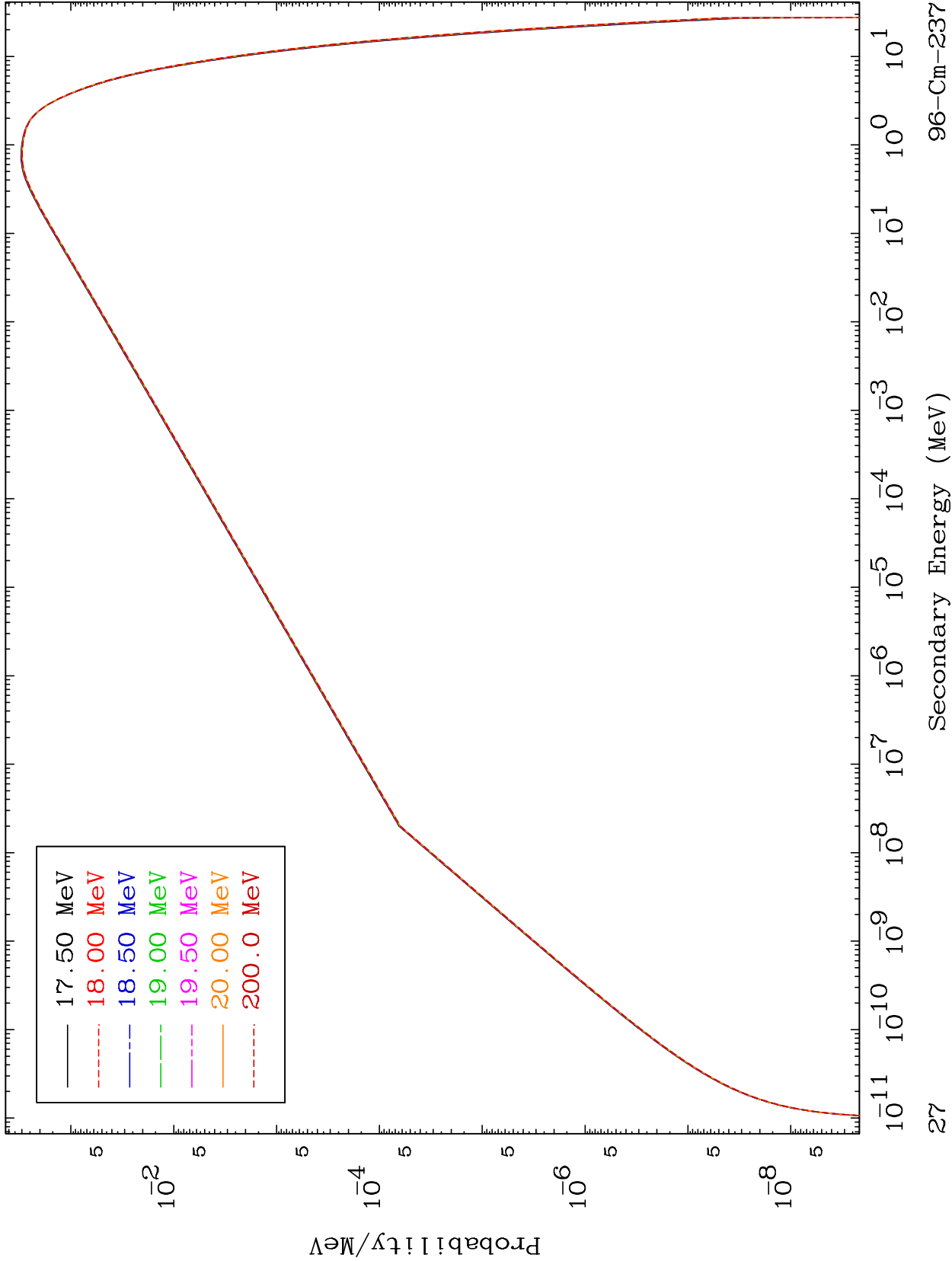
Secondary Energy (MeV)

25

Fission Energy Distributions



Fission Energy Distributions

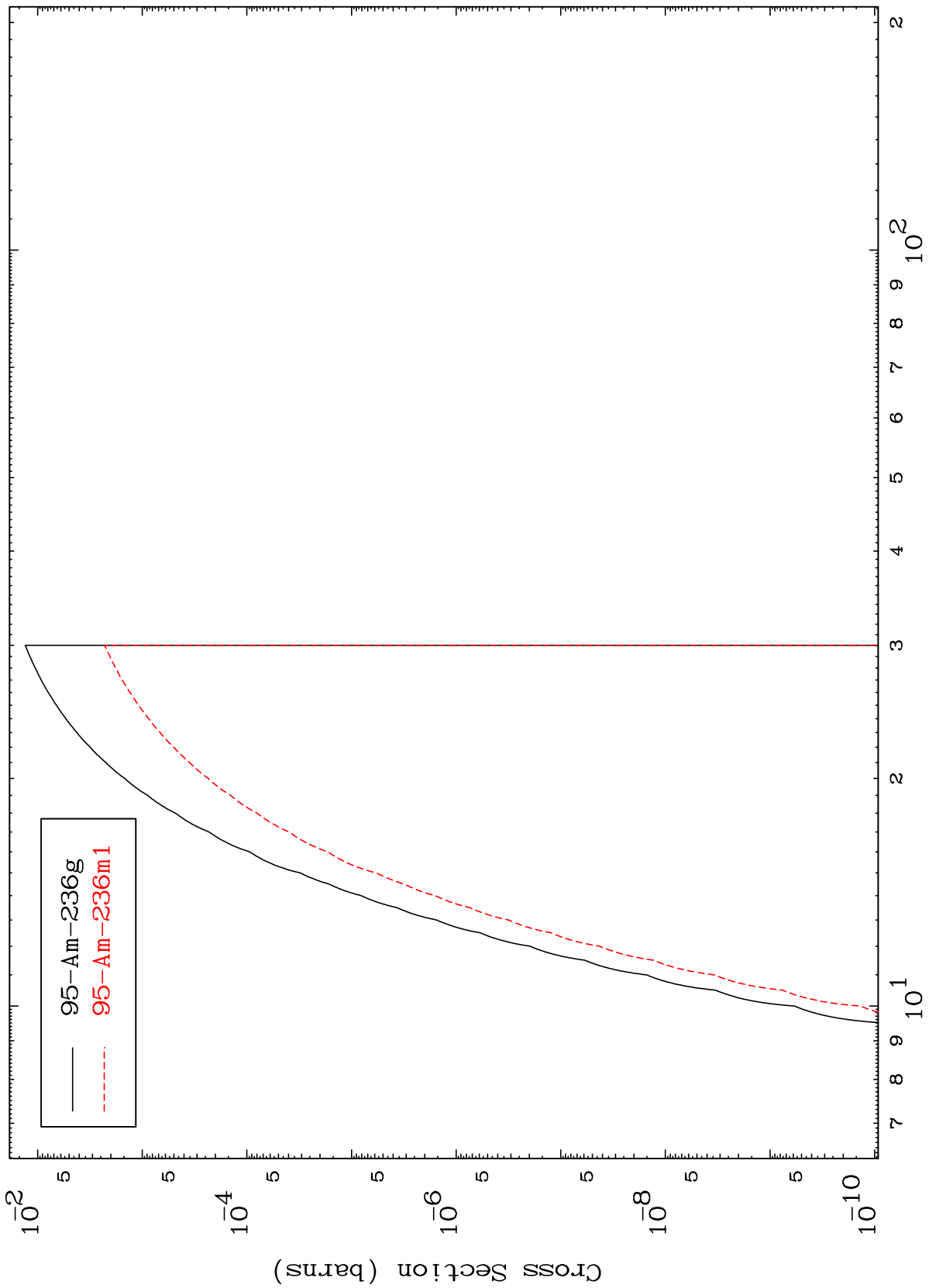


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(n,n') p

96-Cm-237

Radionuclide Production Cross Section



95-Am-236g
95-Am-236m1

Incident Energy (MeV)

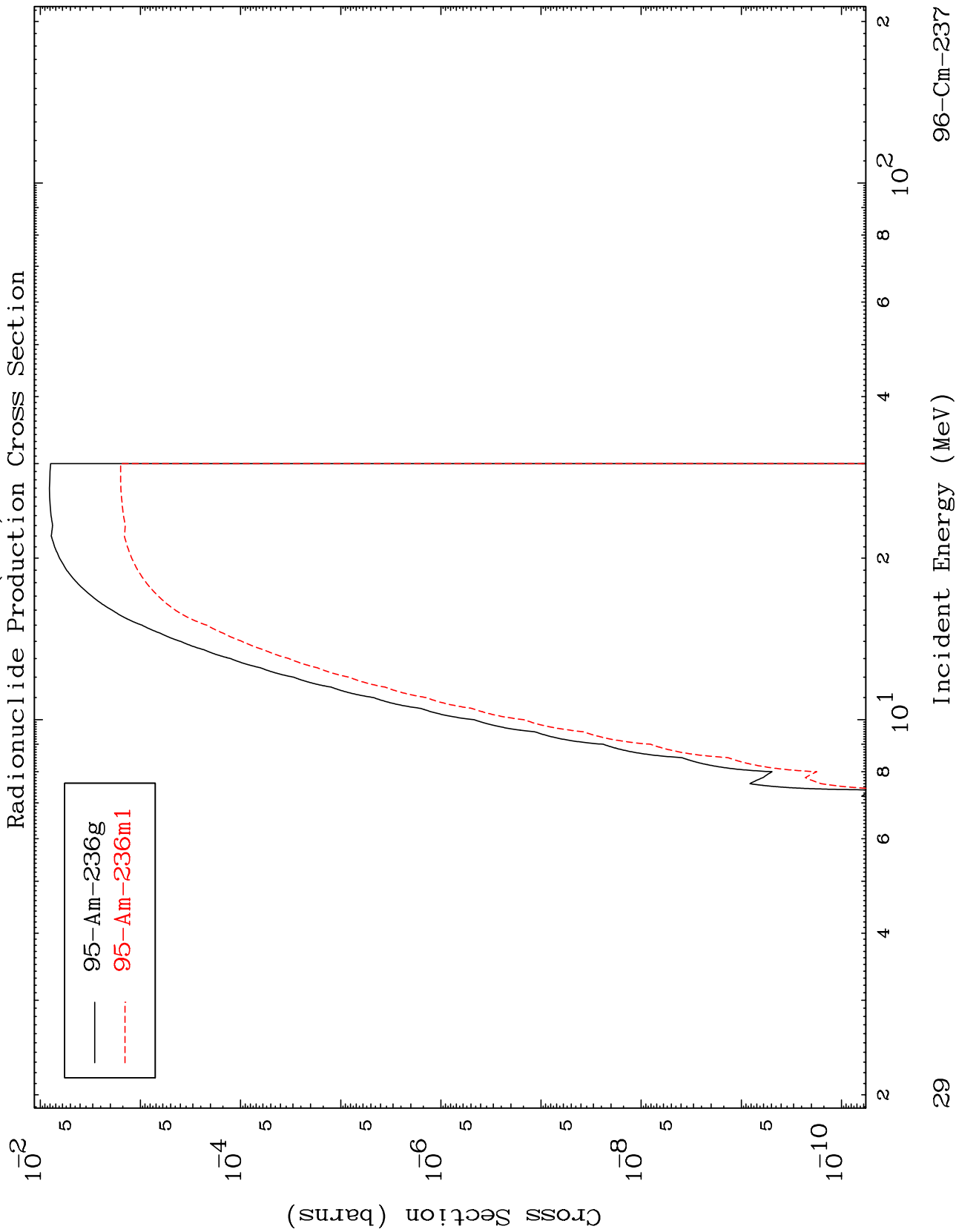
96-Cm-237

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(n,d)

96-Cm-237



29

96-Cm-237