

Program EVALPLOT  
(Version 2021-1)

by

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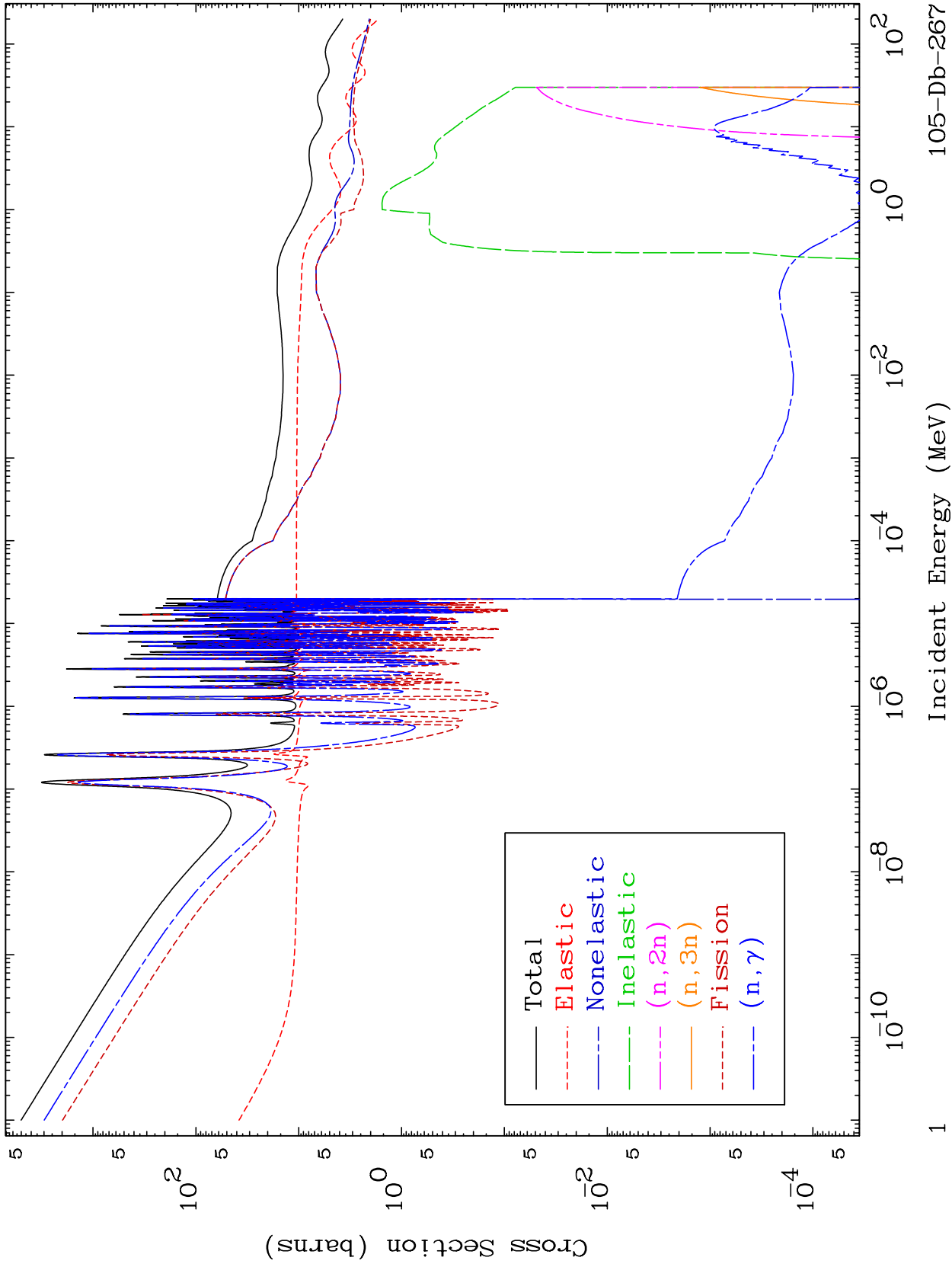
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Press Mouse Button to Start

MAT 567

Neutron Major  
293 Kelvin Cross Sections

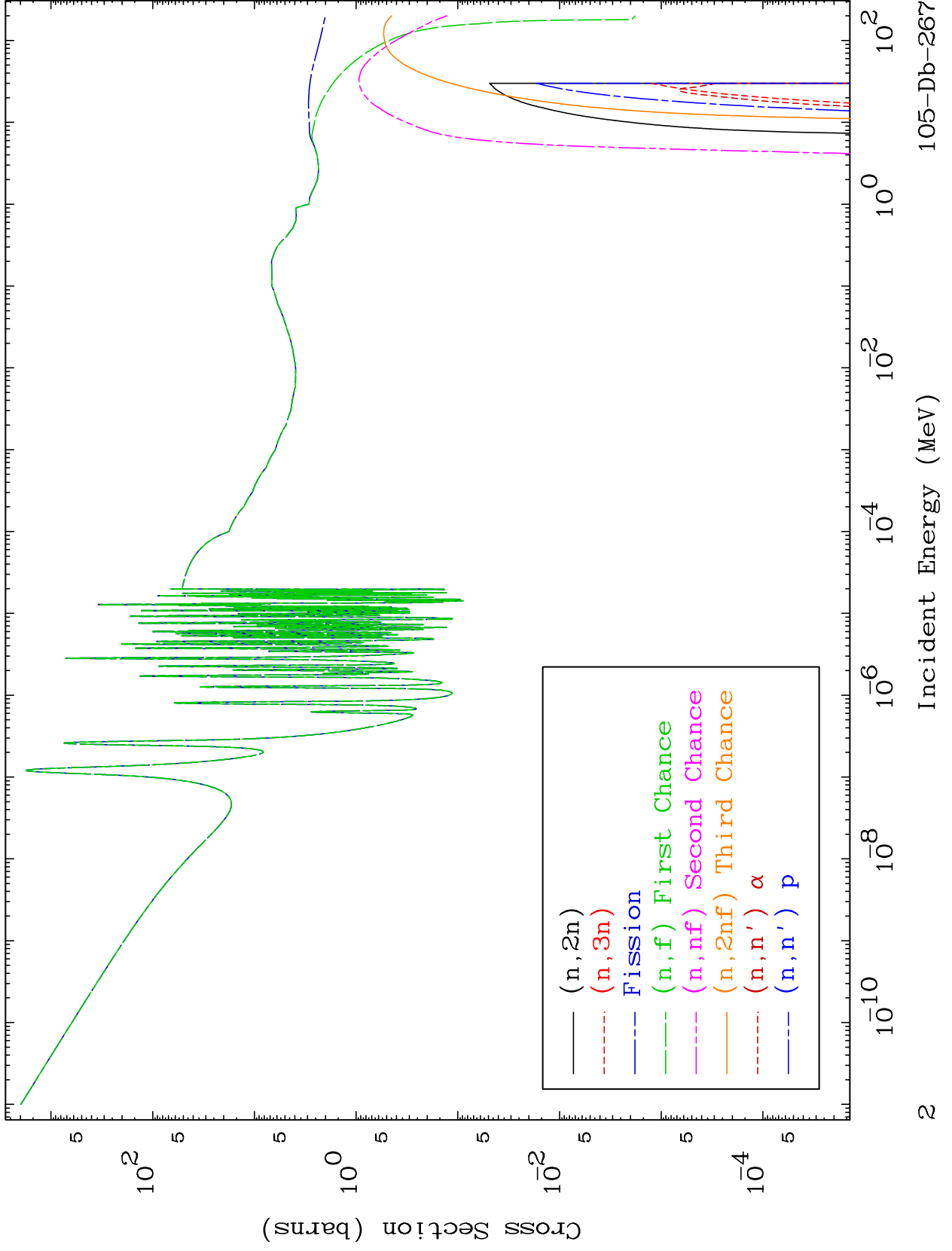
105-Db-267

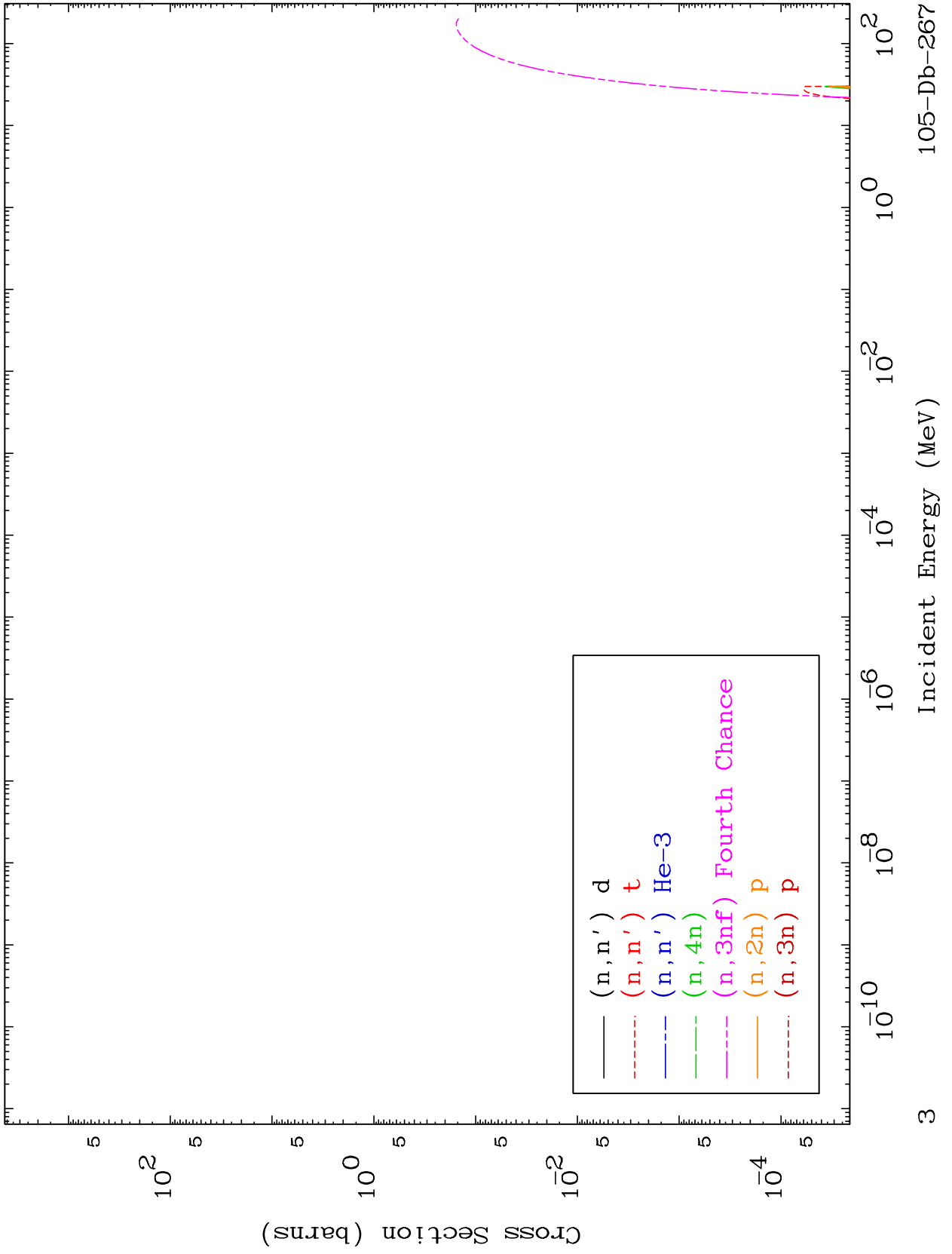


MAT 567

Neutron Absorption  
293 Kelvin Cross Sections

105-Db-267

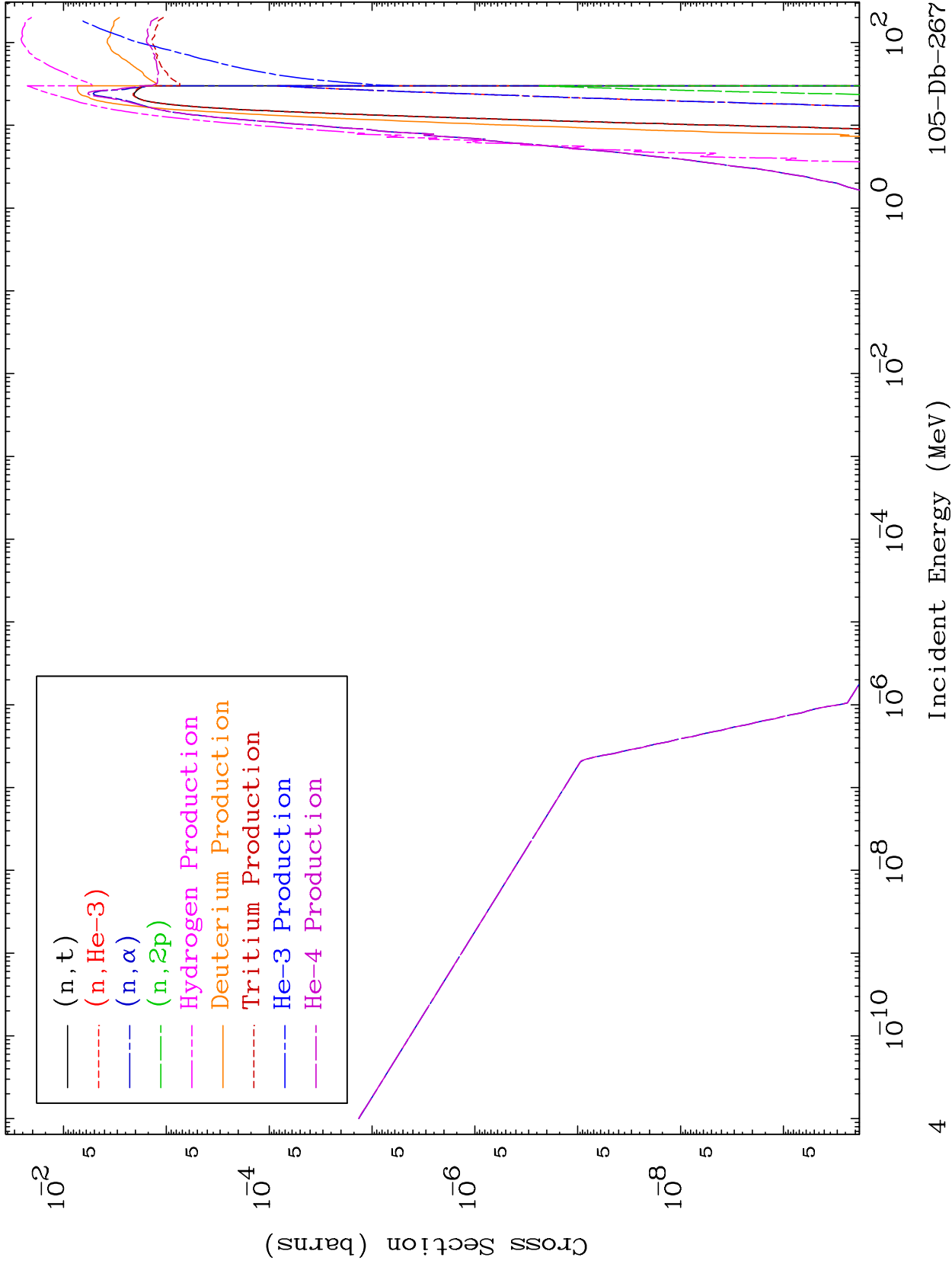




MAT 567

Neutron Absorption  
293 Kelvin Cross Sections

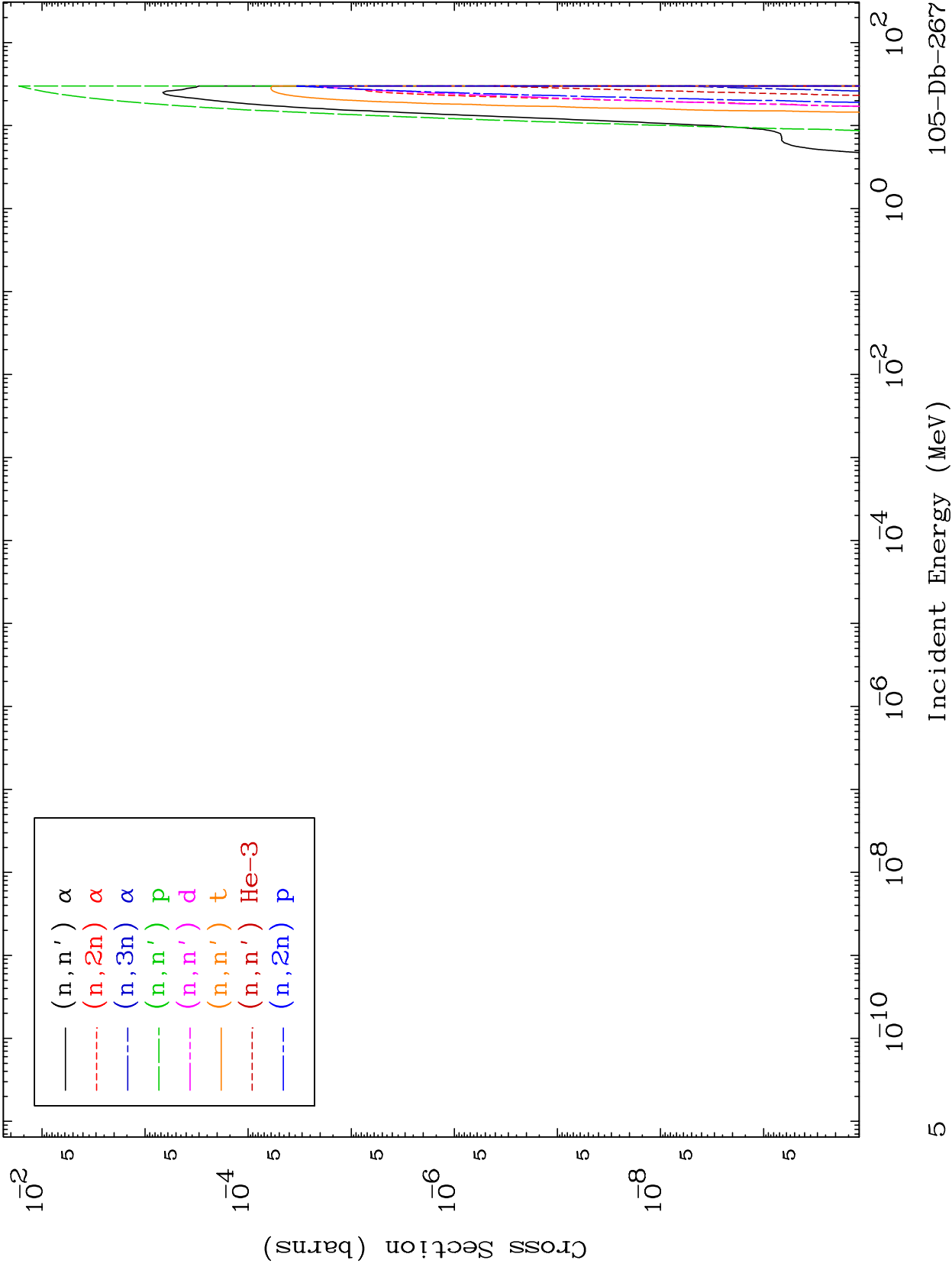
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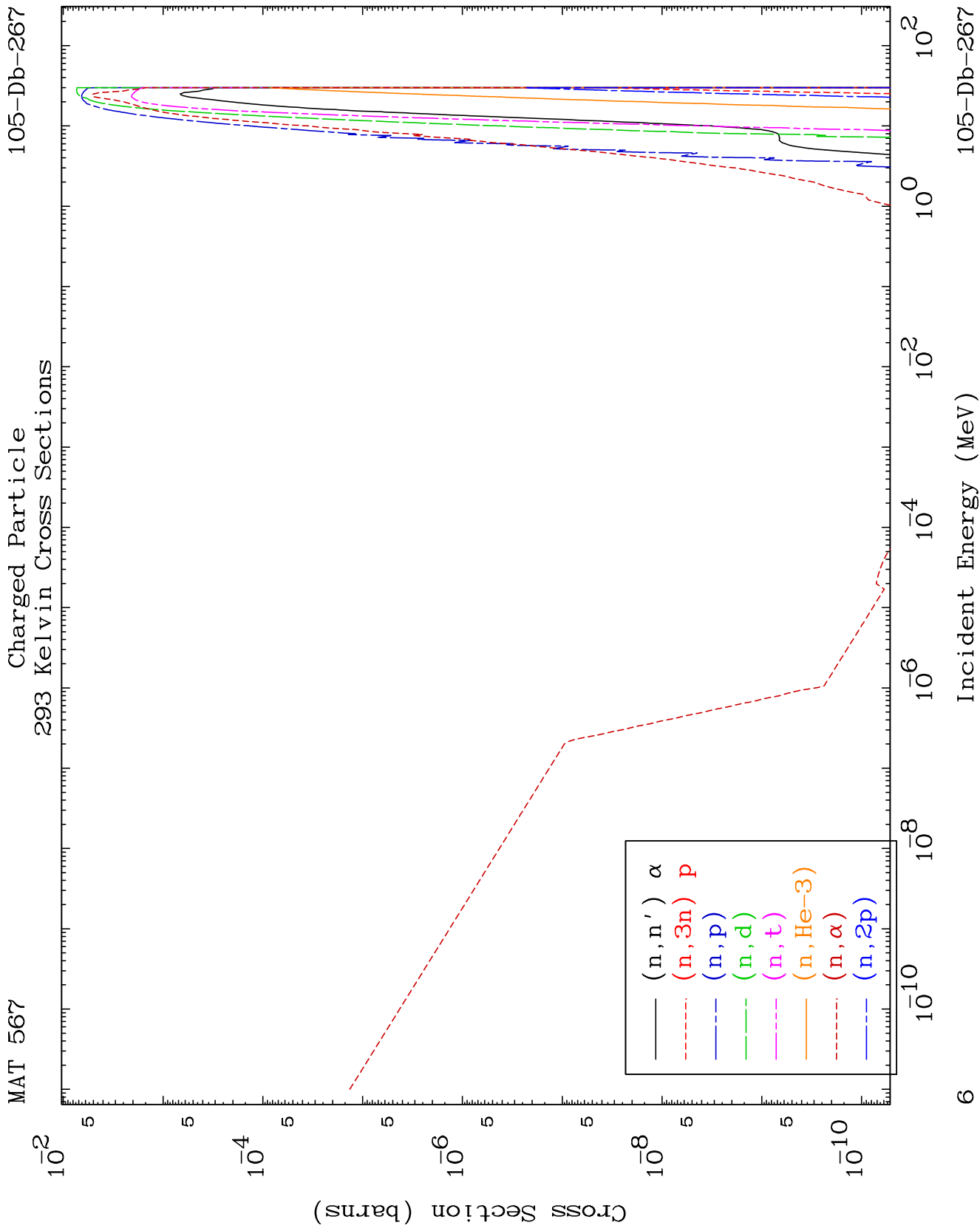


MAT 567

Charged Particle  
293 Kelvin Cross Sections

105-Db-267

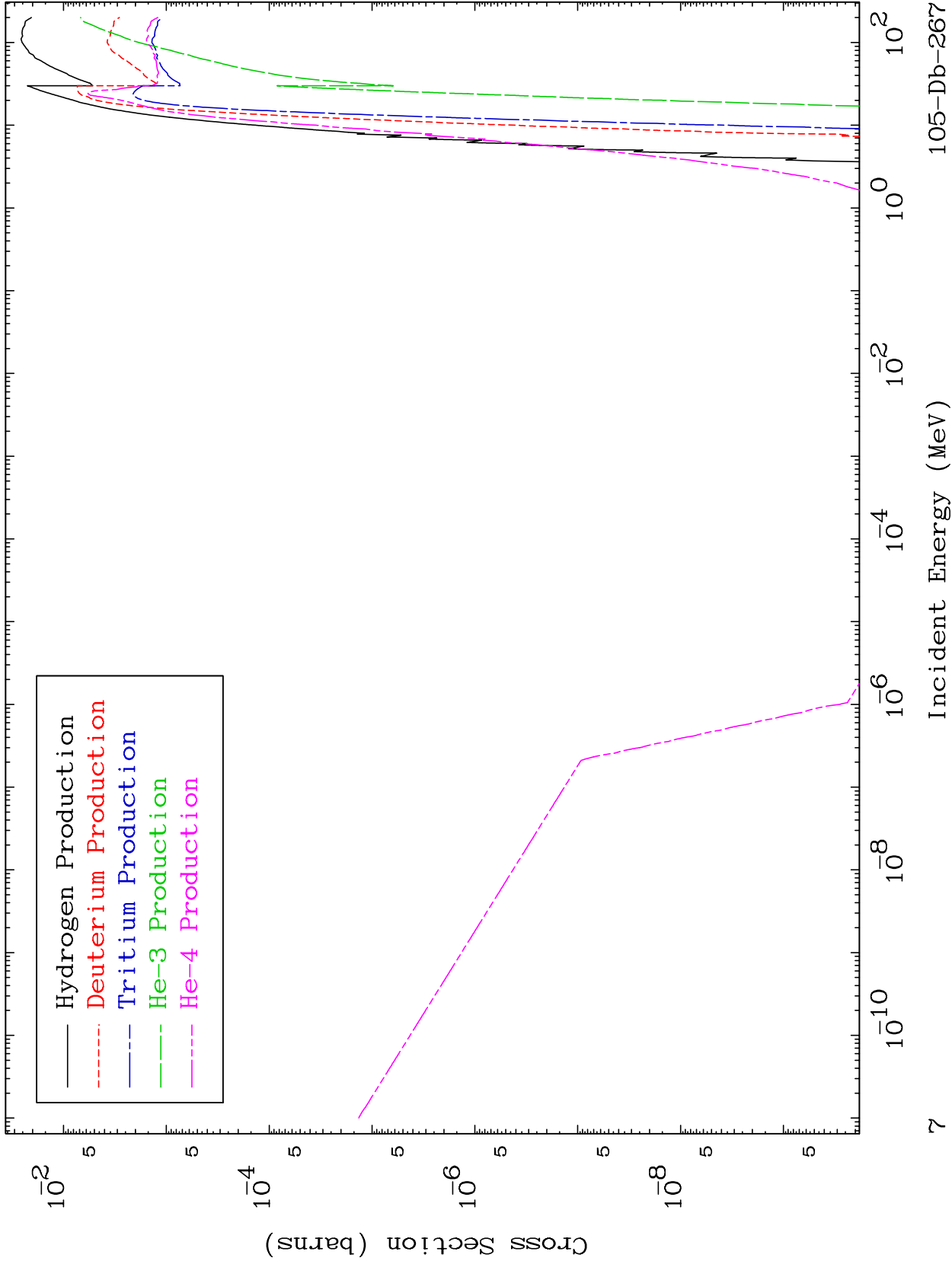




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Particle Production  
293 Kelvin Cross Sections

105-Db-267

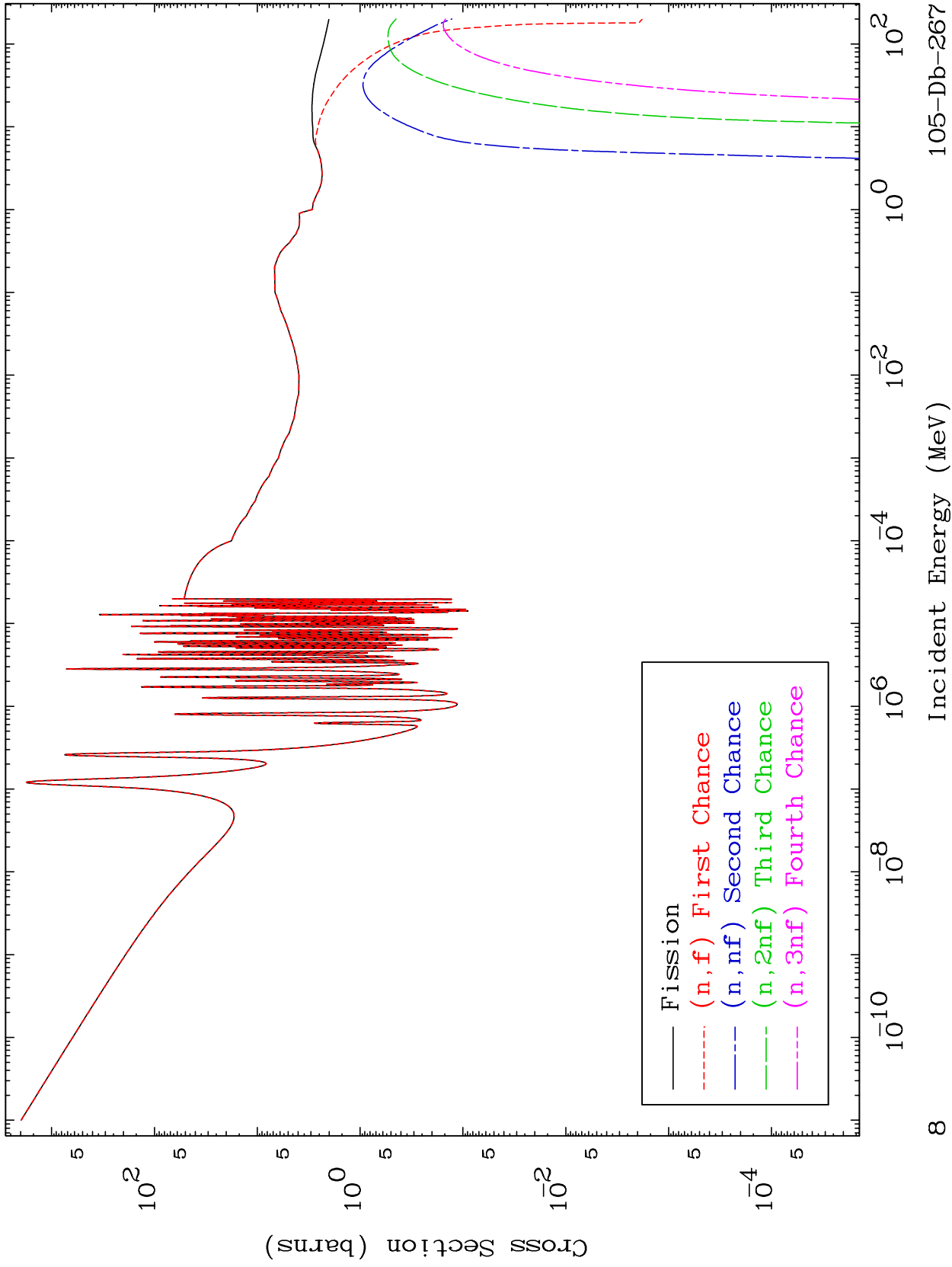


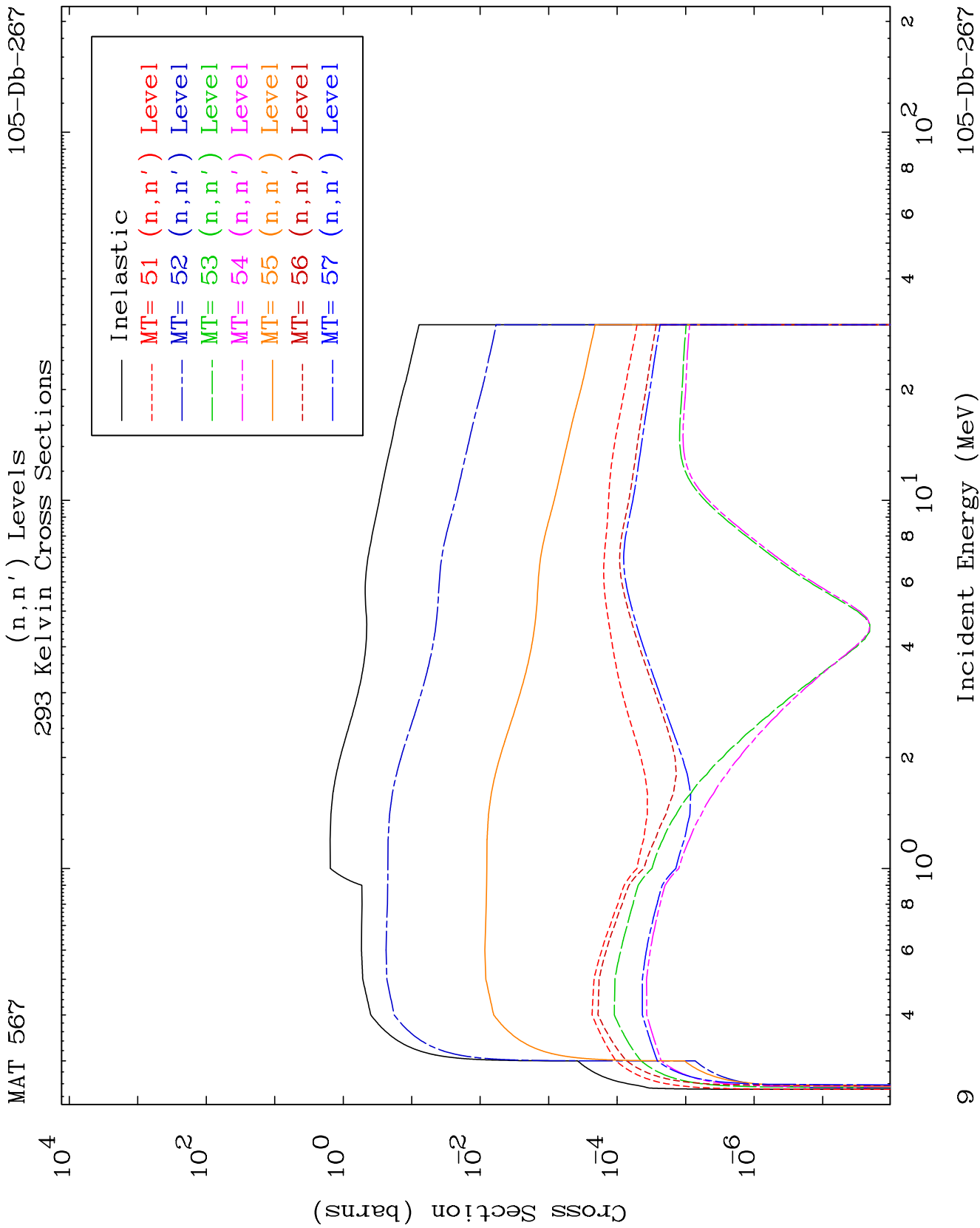


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Fission  
293 Kelvin Cross Sections

105-Db-267

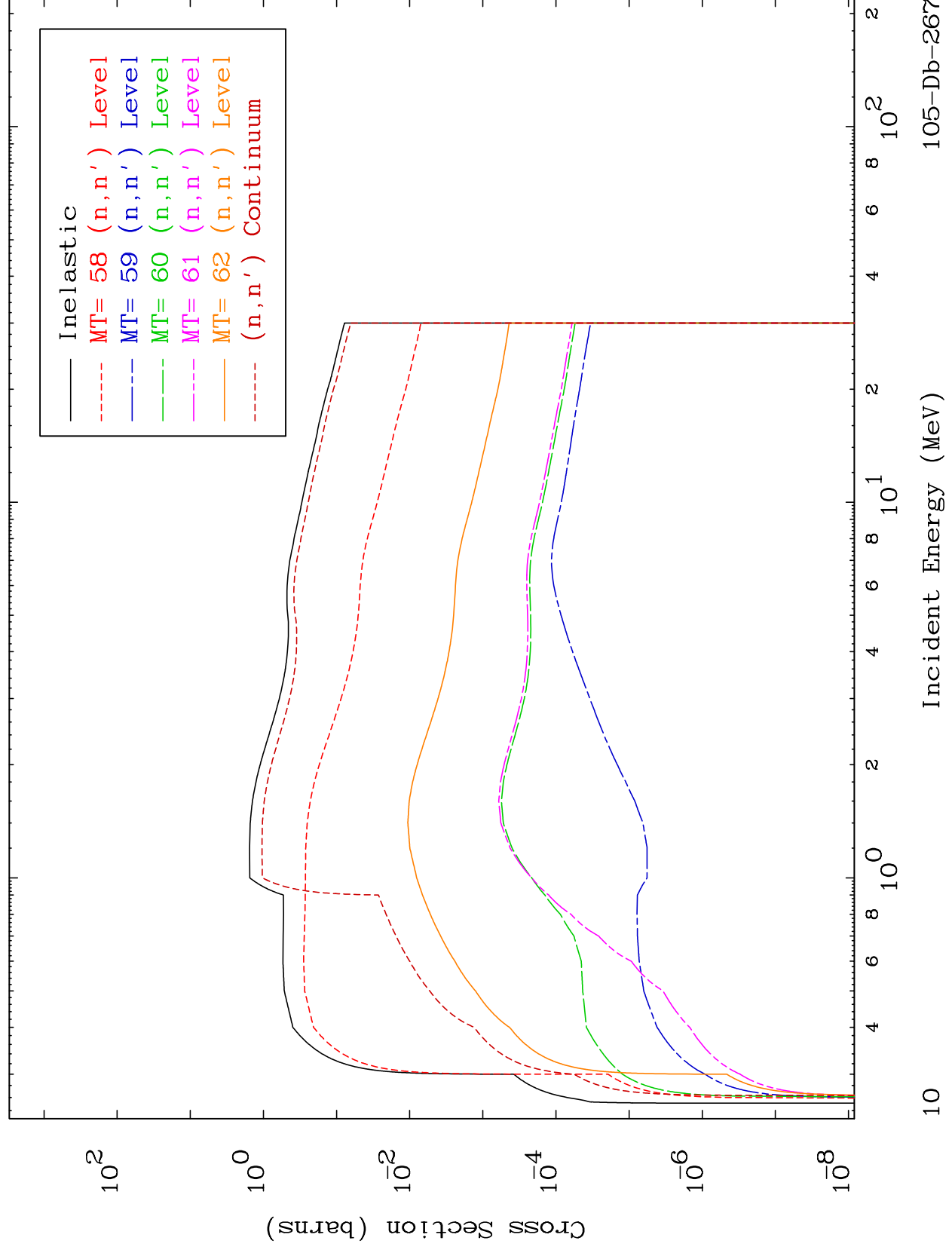




MAT 567

(n,n') Levels  
293 Kelvin Cross Sections

105-Db-267



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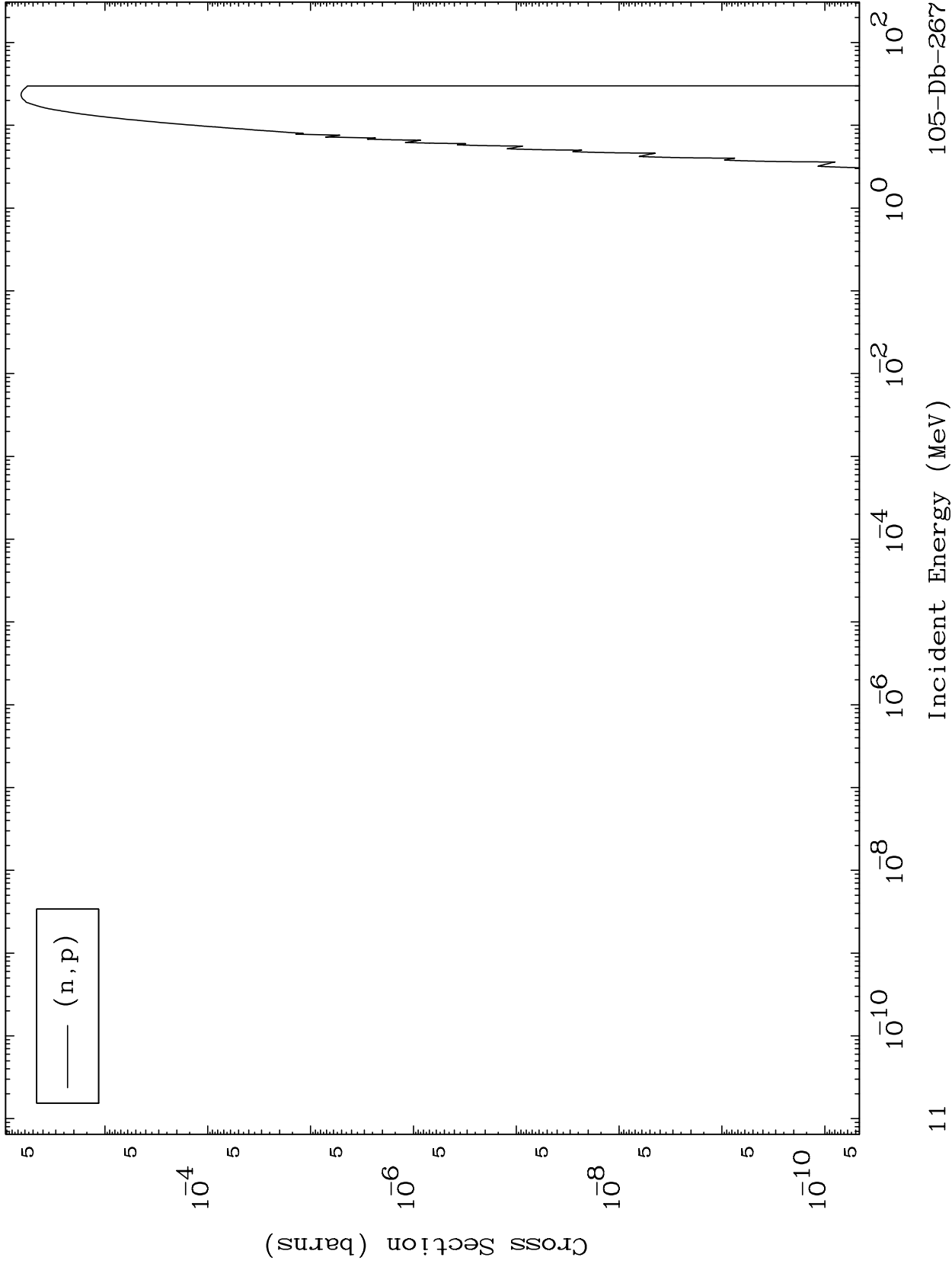
Incident Energy (MeV)

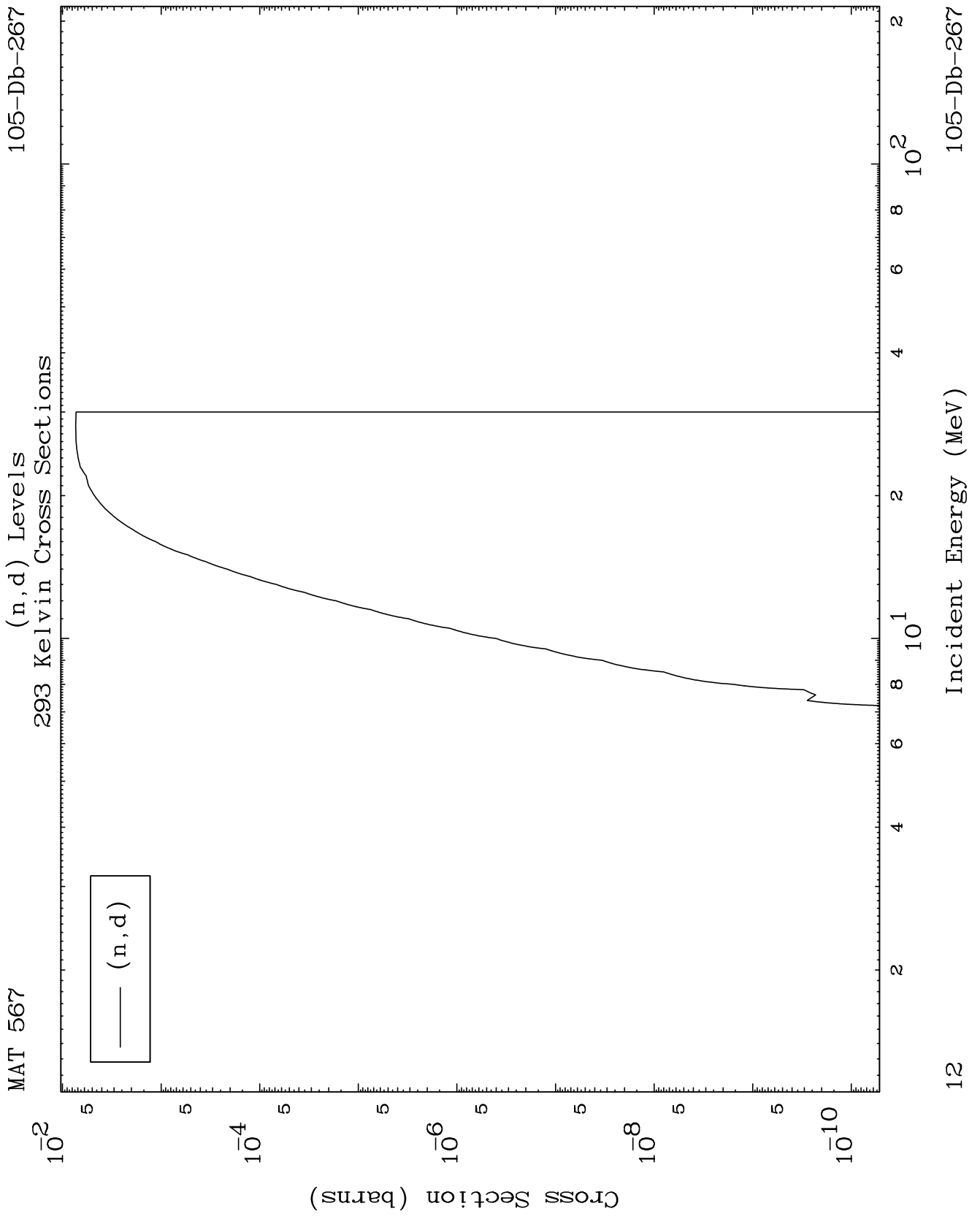
105-Db-267

MAT 567

(n,p) Levels  
293 Kelvin Cross Sections

105-Db-267

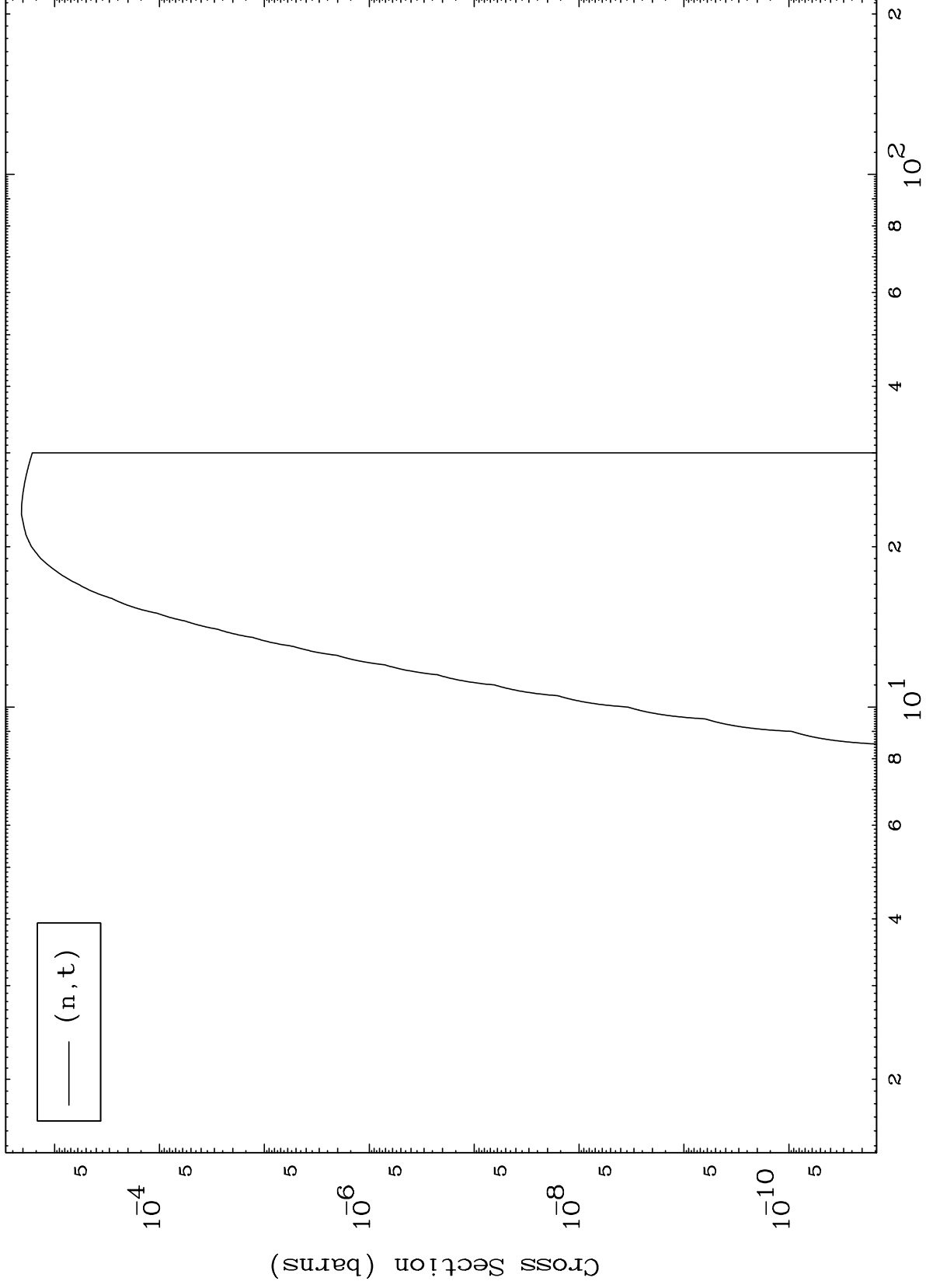




MAT 567

(n,t) Levels  
293 Kelvin Cross Sections

105-Db-267



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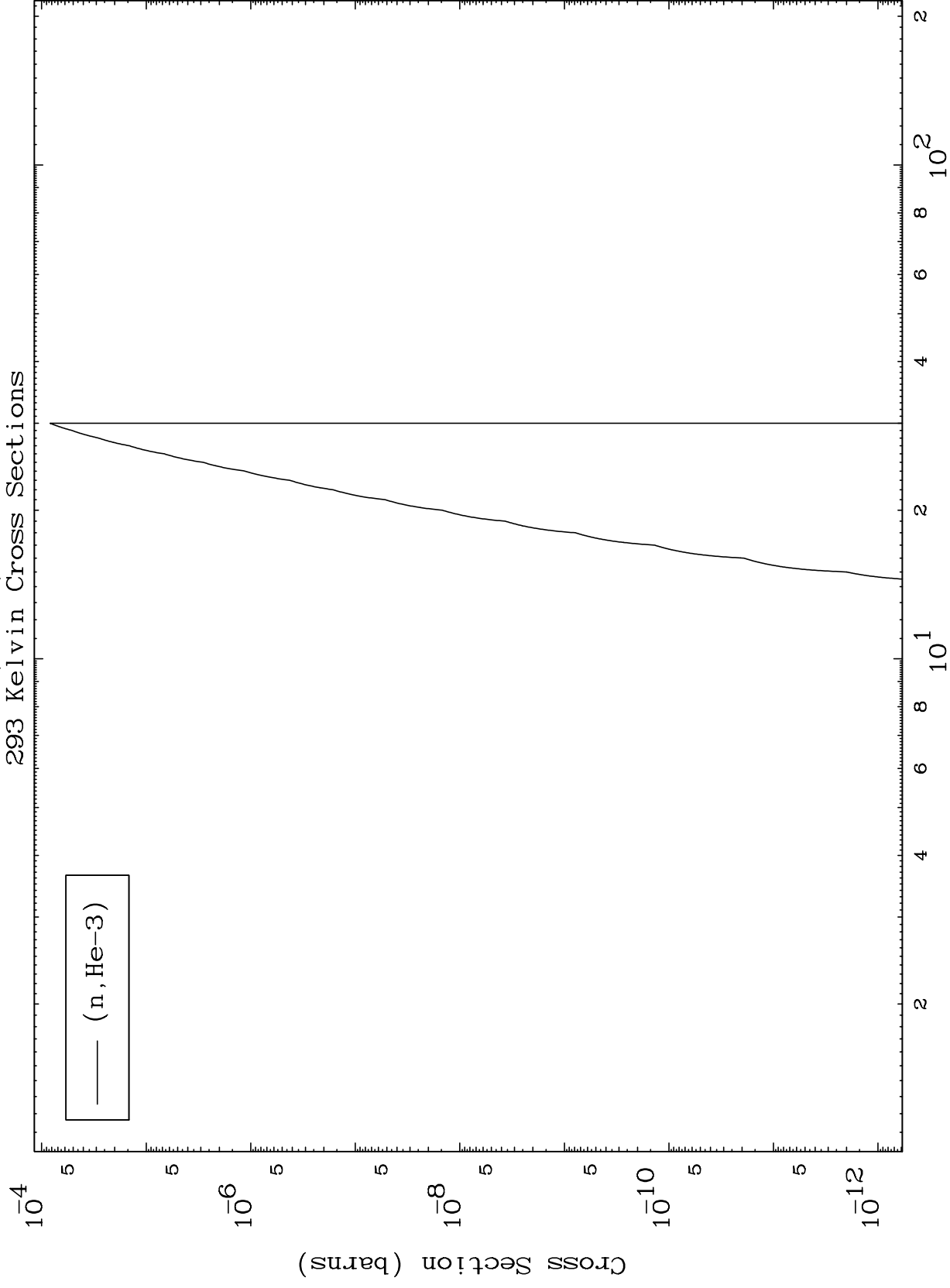
Incident Energy (MeV)

105-Db-267

MAT 567

(n,He3) Levels  
293 Kelvin Cross Sections

105-Db-267



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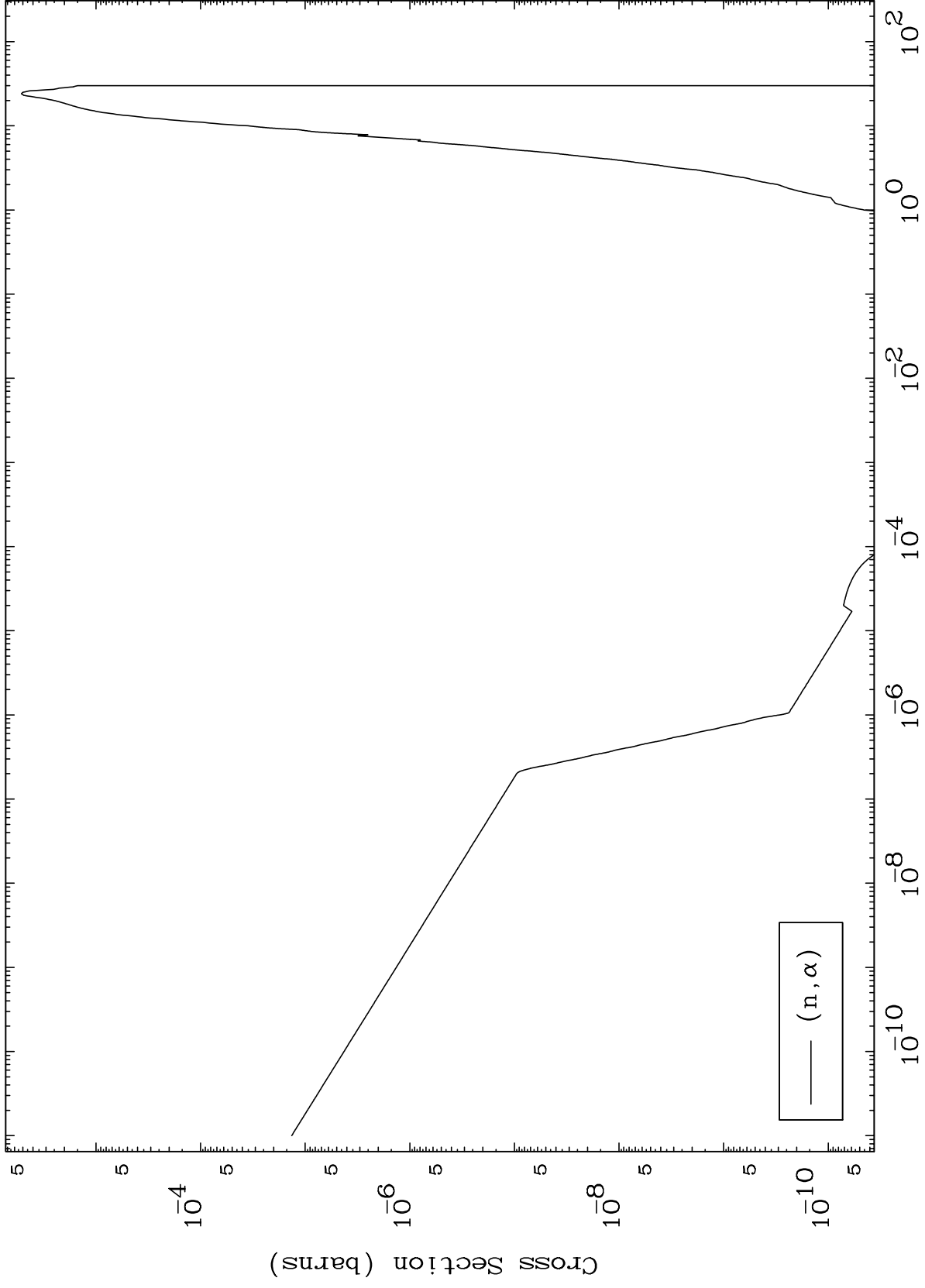
Incident Energy (MeV)

105-Db-267

MAT 567

(n,α) Levels  
293 Kelvin Cross Sections

105-Db-267



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Incident Energy (MeV)

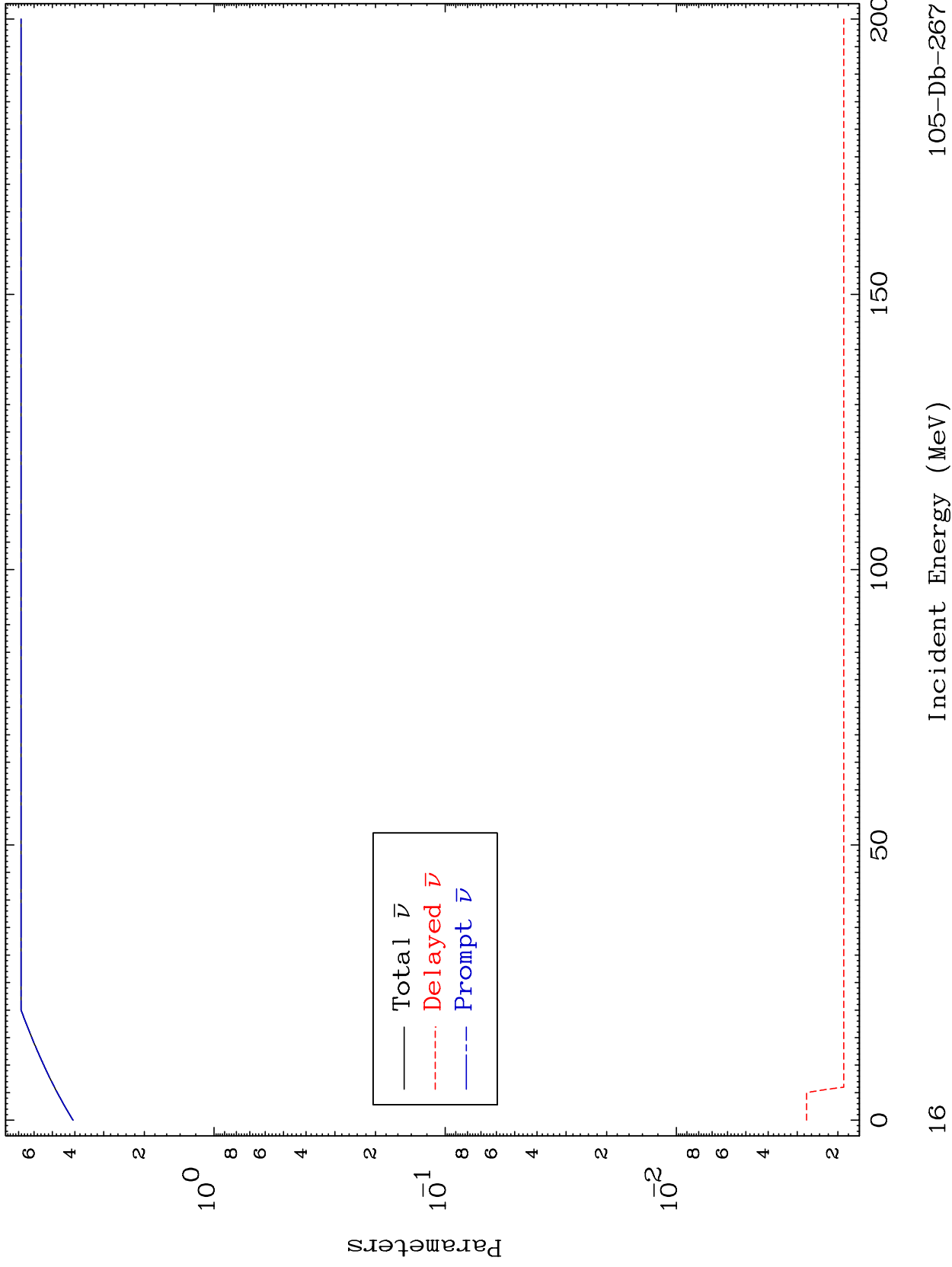
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Energy Release  
Parameters

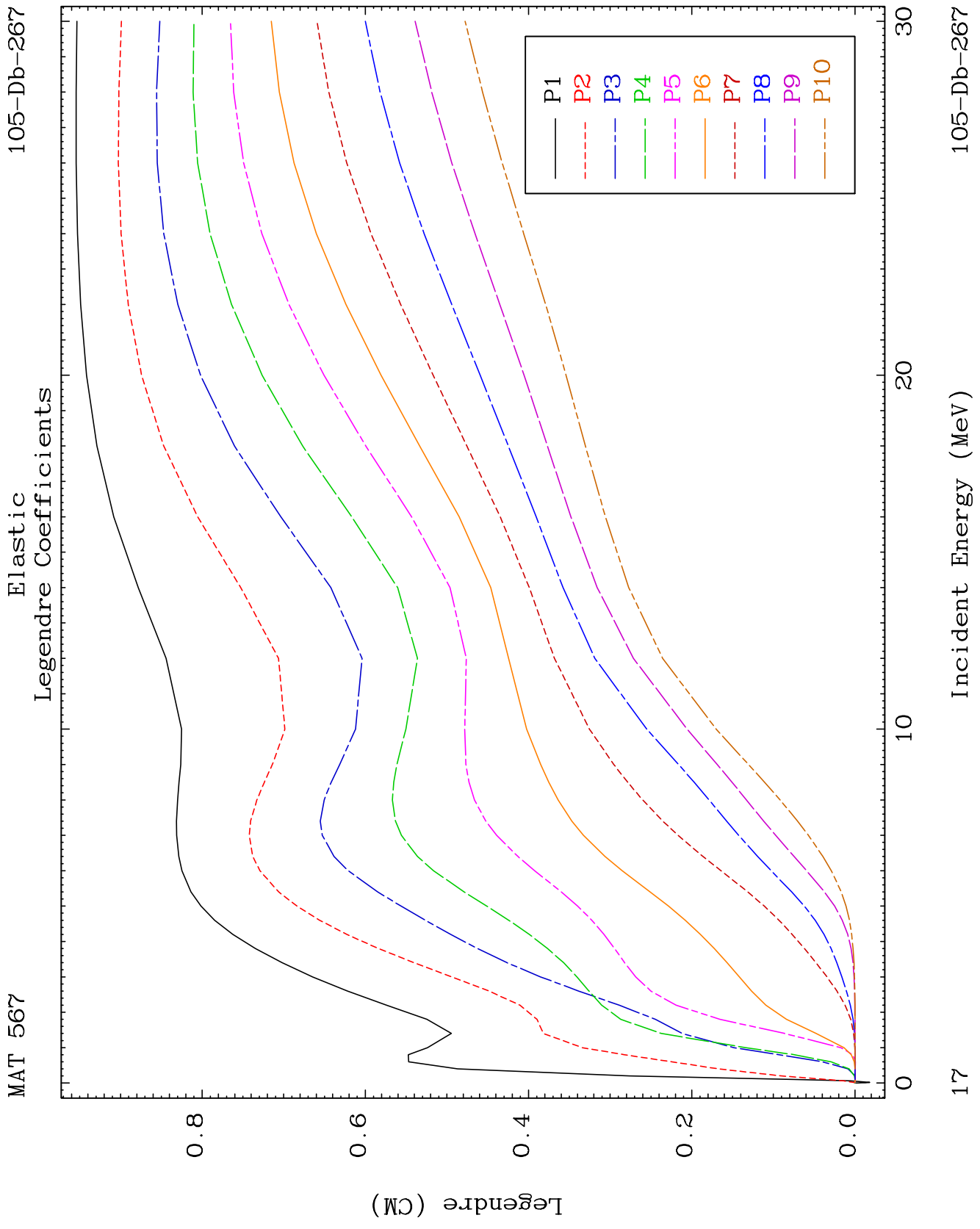
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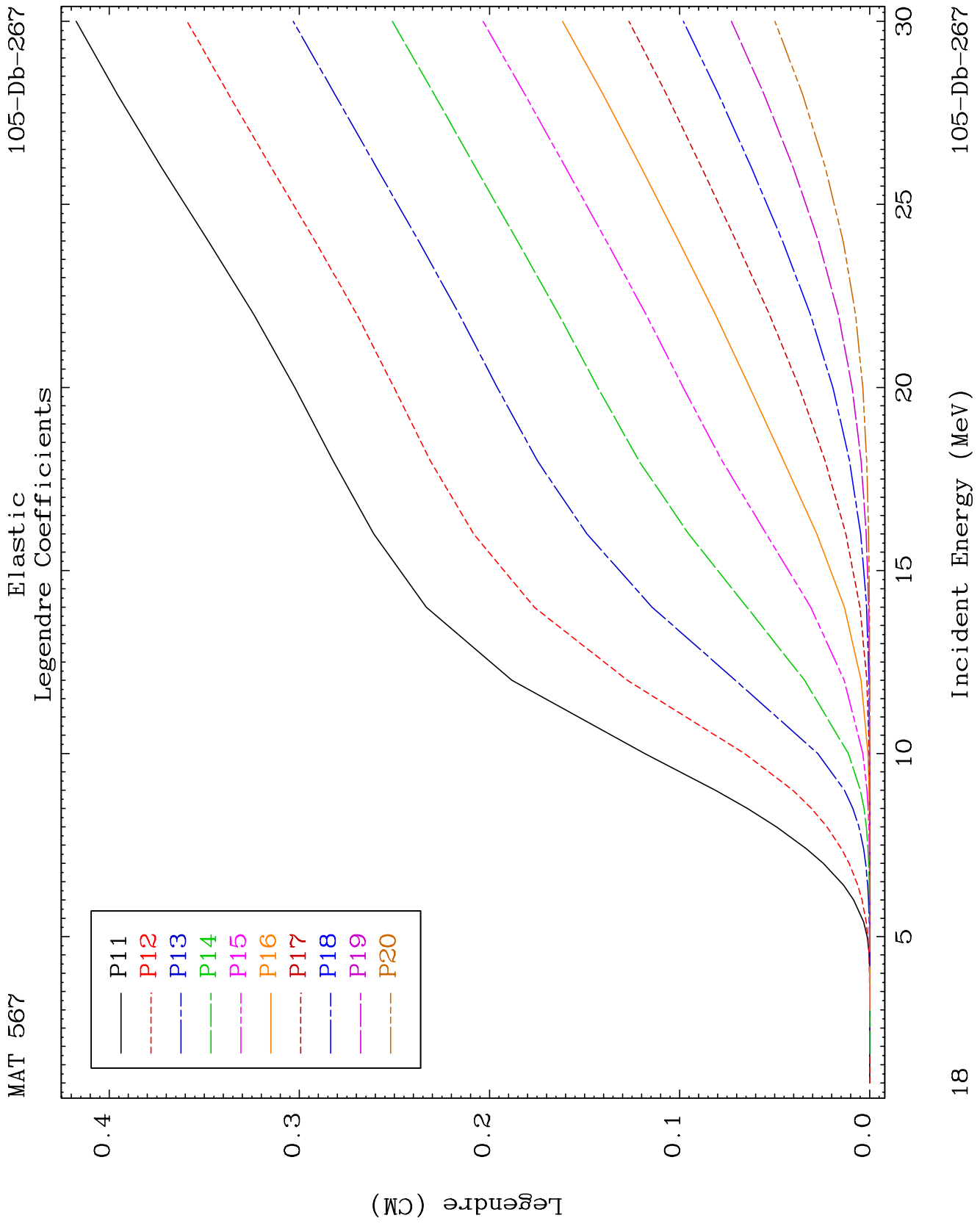


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Incident Energy (MeV)

105-Db-267

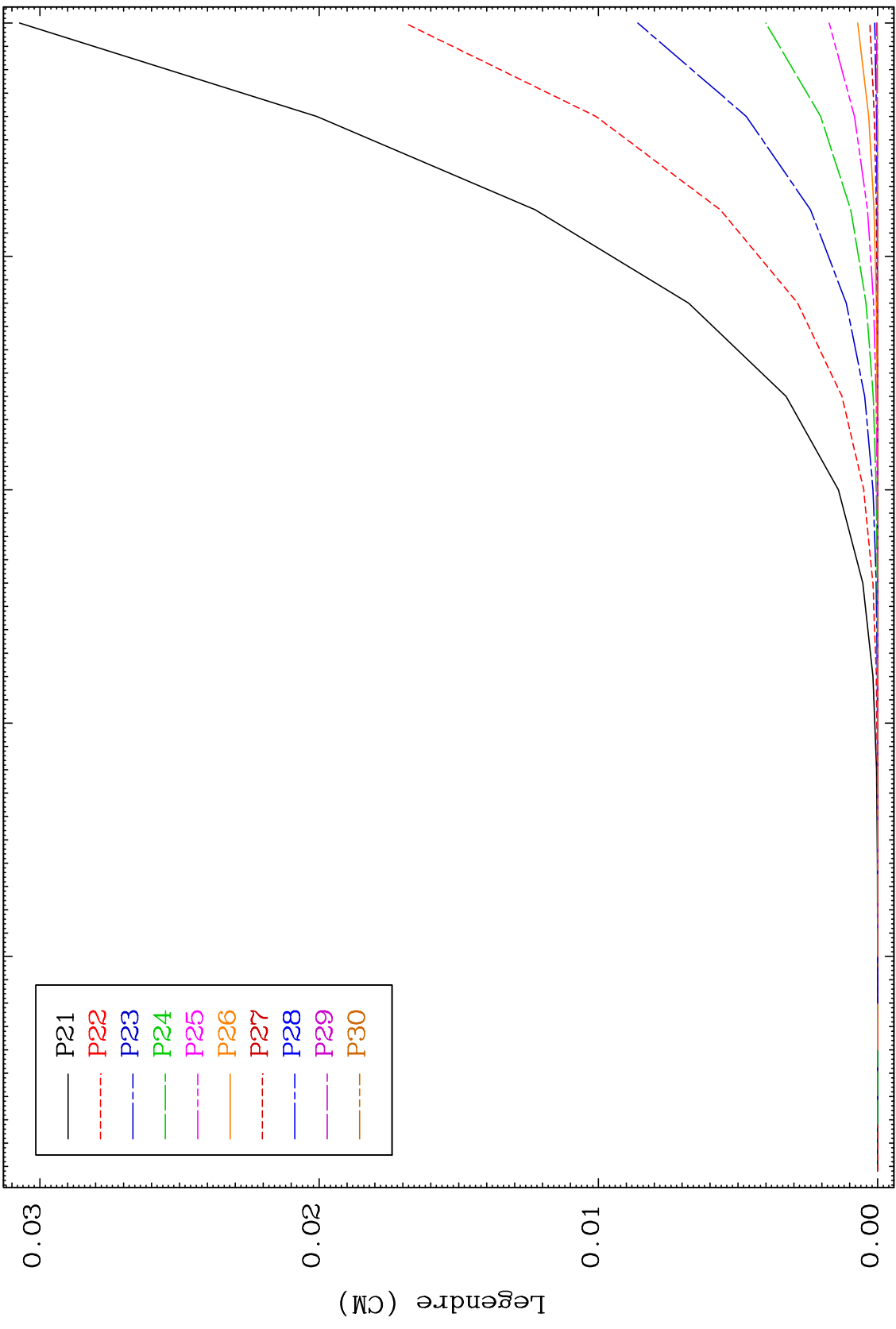




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Elastic Legendre Coefficients

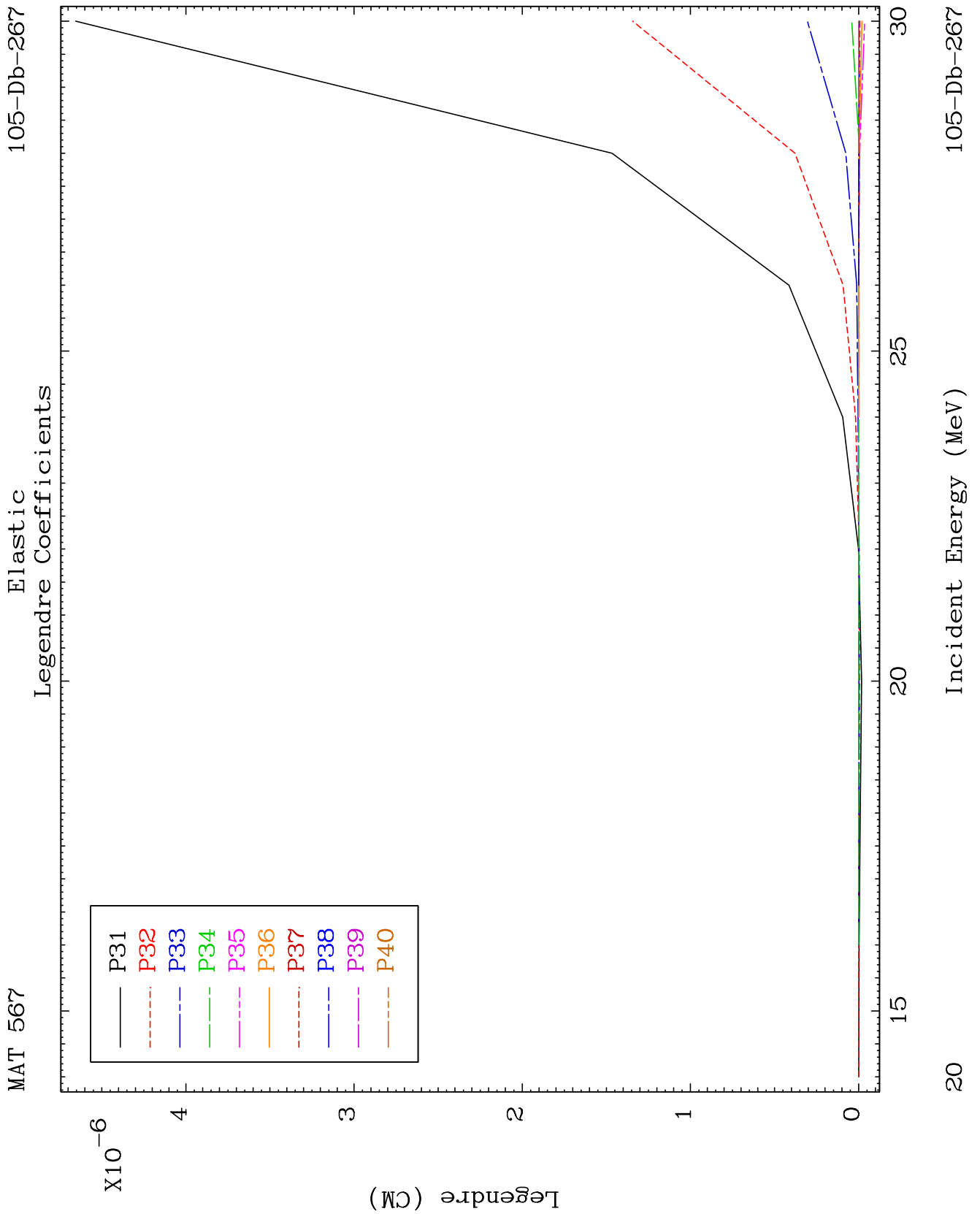
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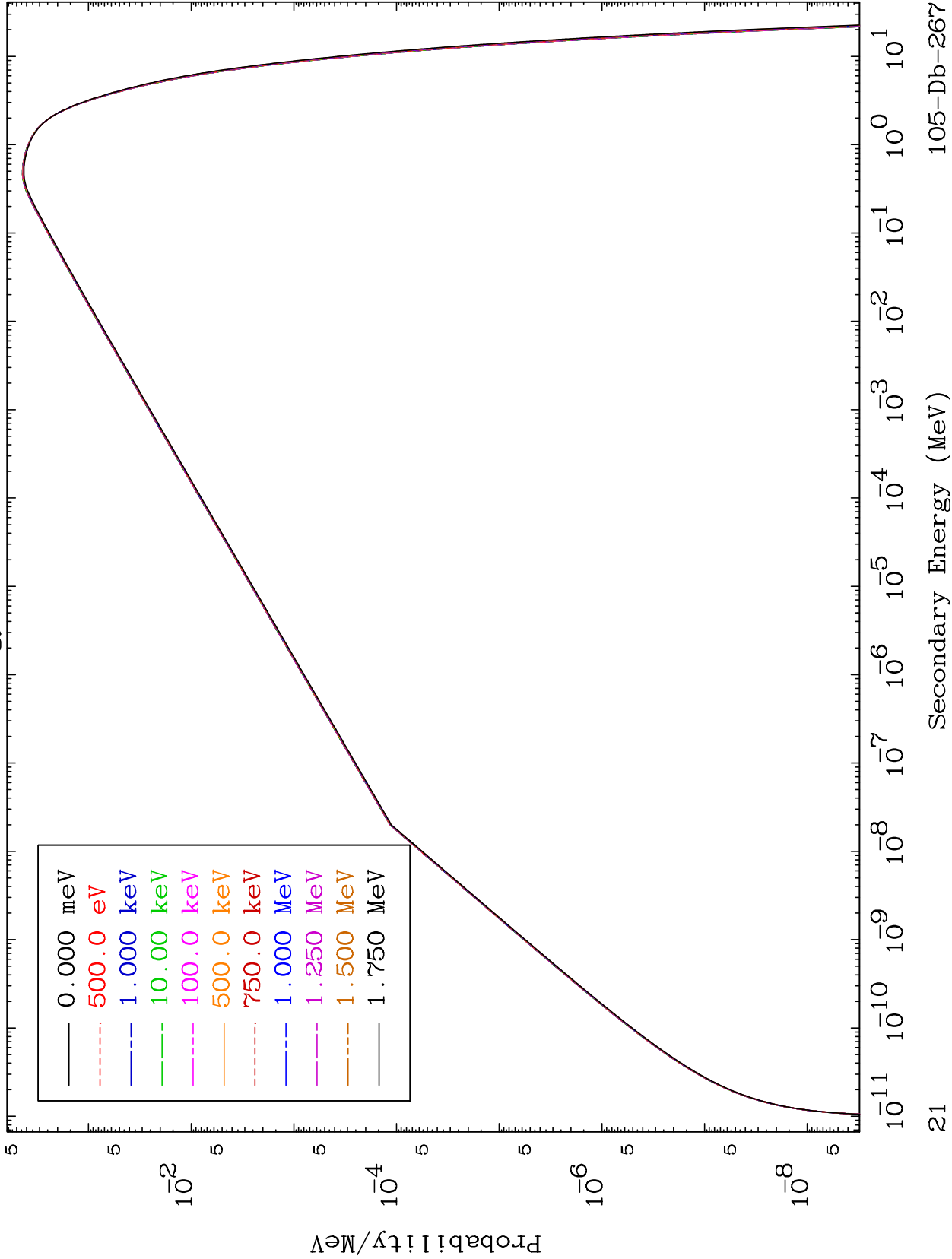
19

Incident Energy (MeV)

105-Db-267



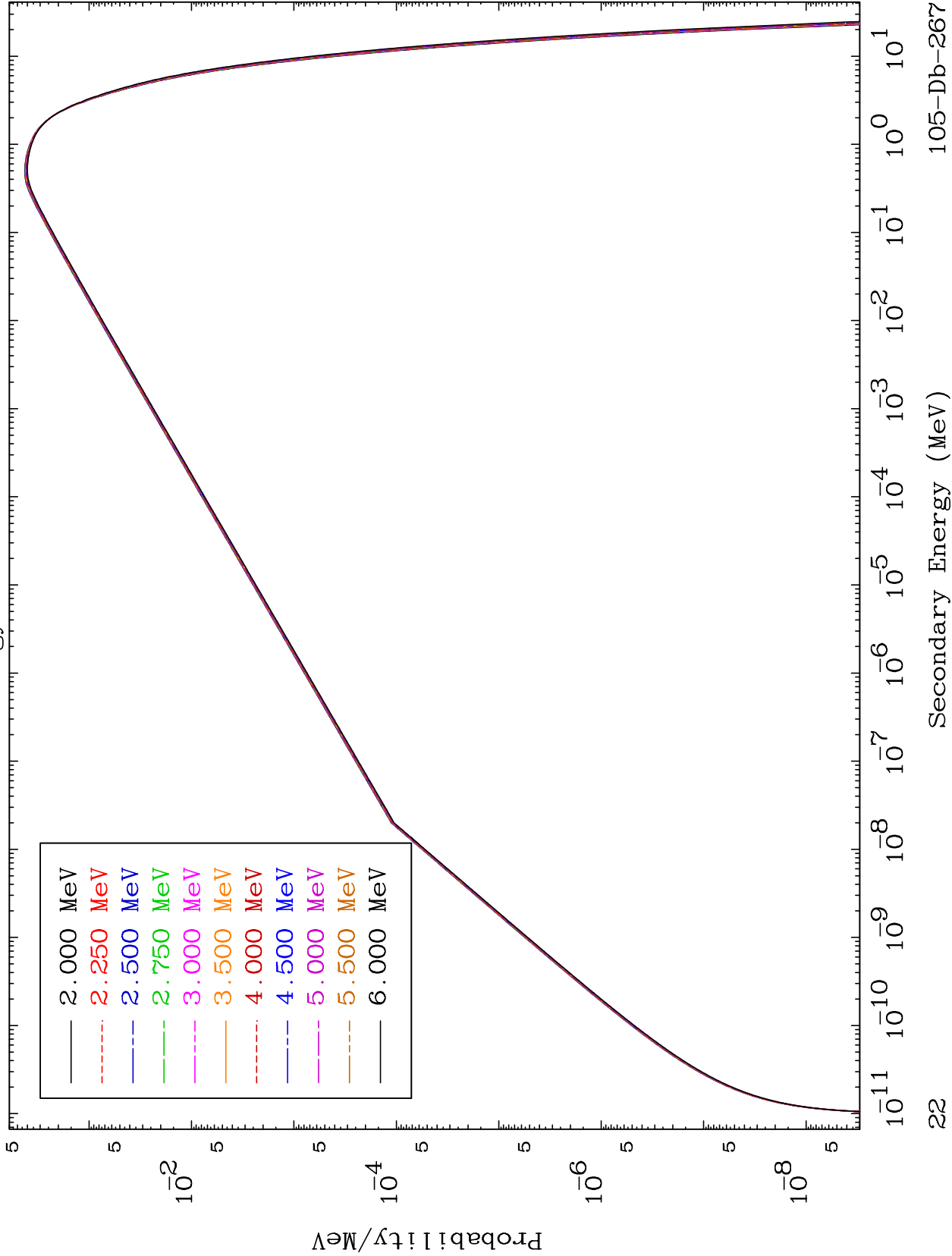
Fission Energy Distributions



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### Fission Energy Distributions

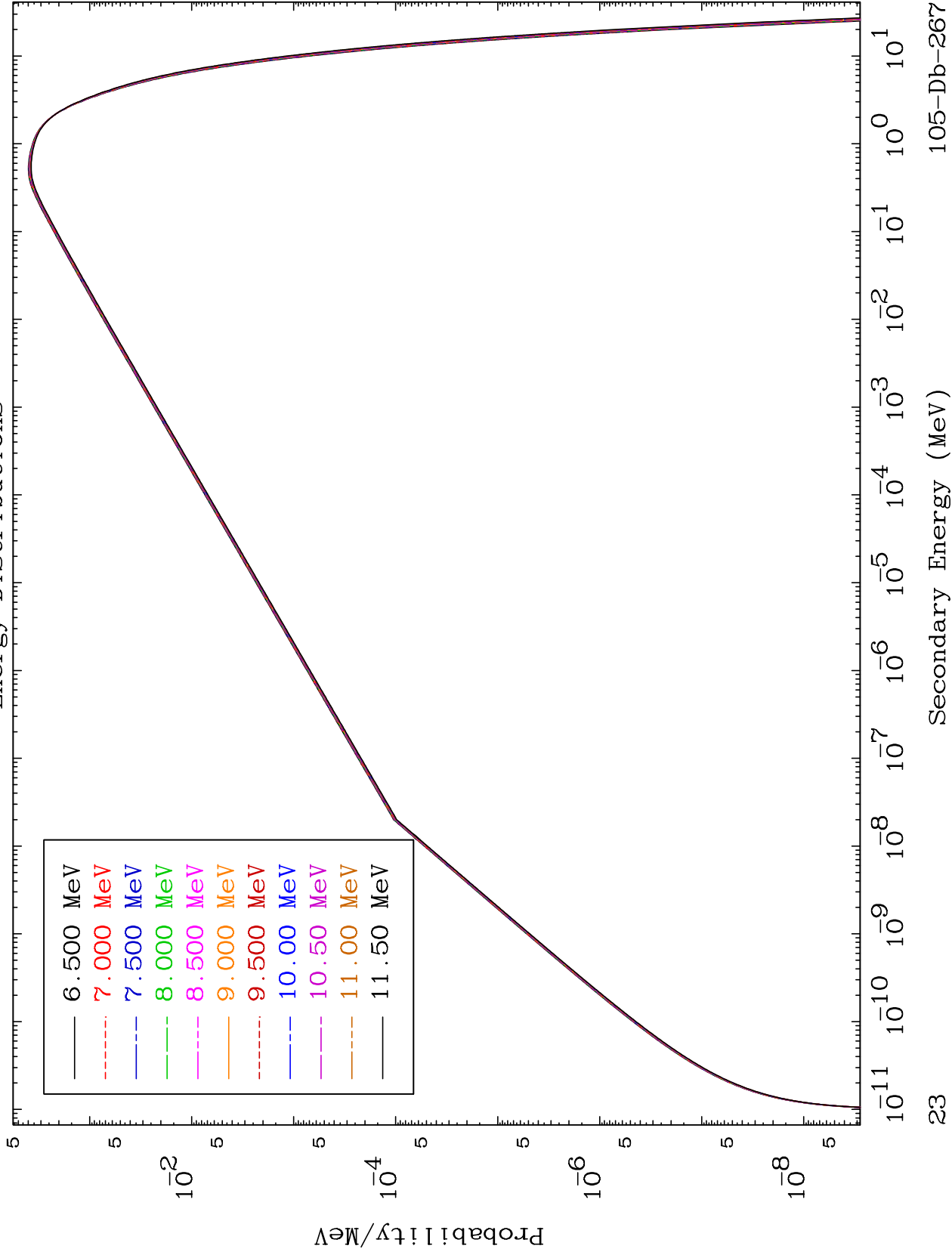
105-Db-267



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### Fission Energy Distributions

105-Db-267



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Secondary Energy (MeV)

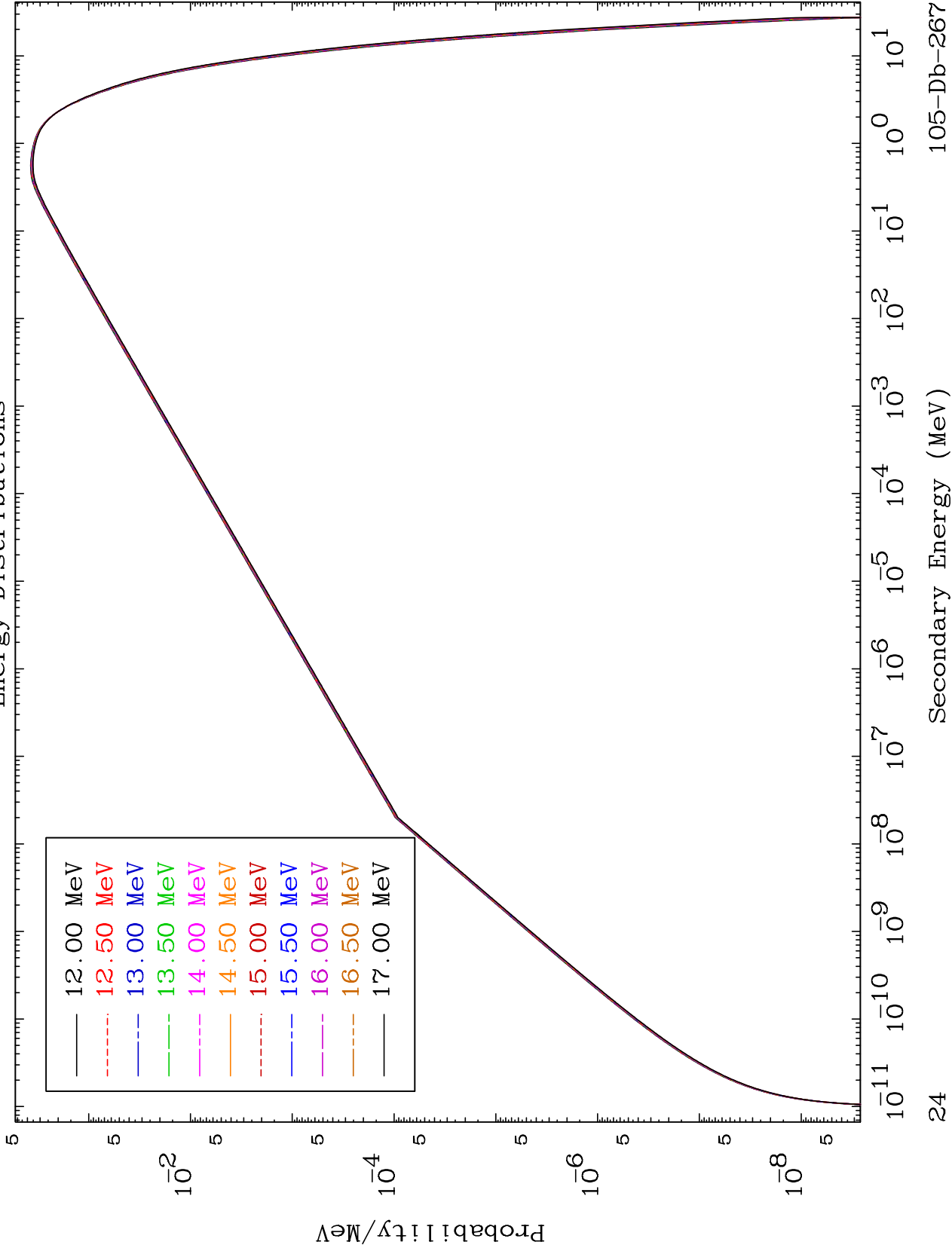
23



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### Fission Energy Distributions

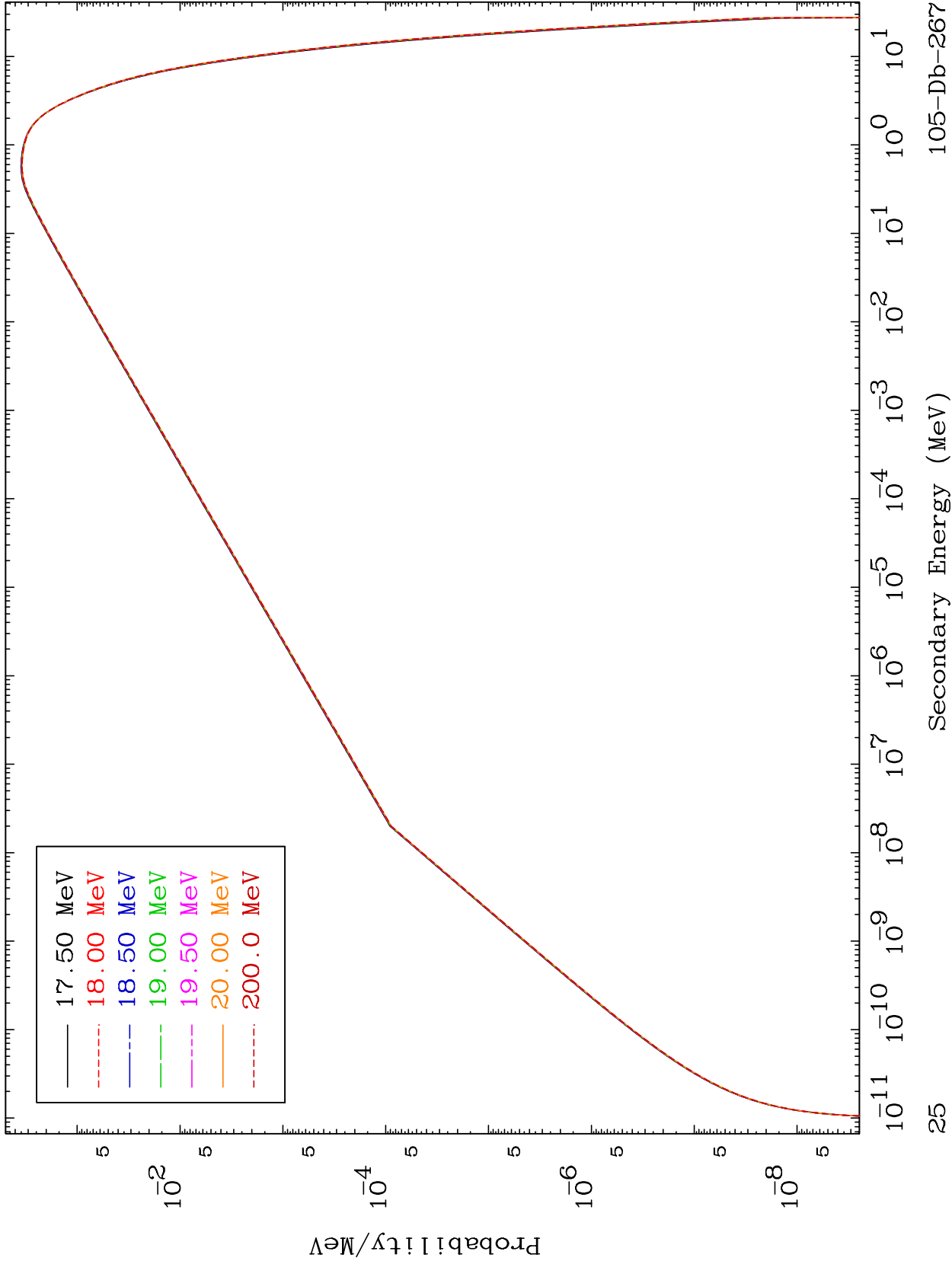
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### Fission Energy Distributions

105-Db-267



105-Db-267

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