

Program EVALPLOT
(Version 2021-1)

by

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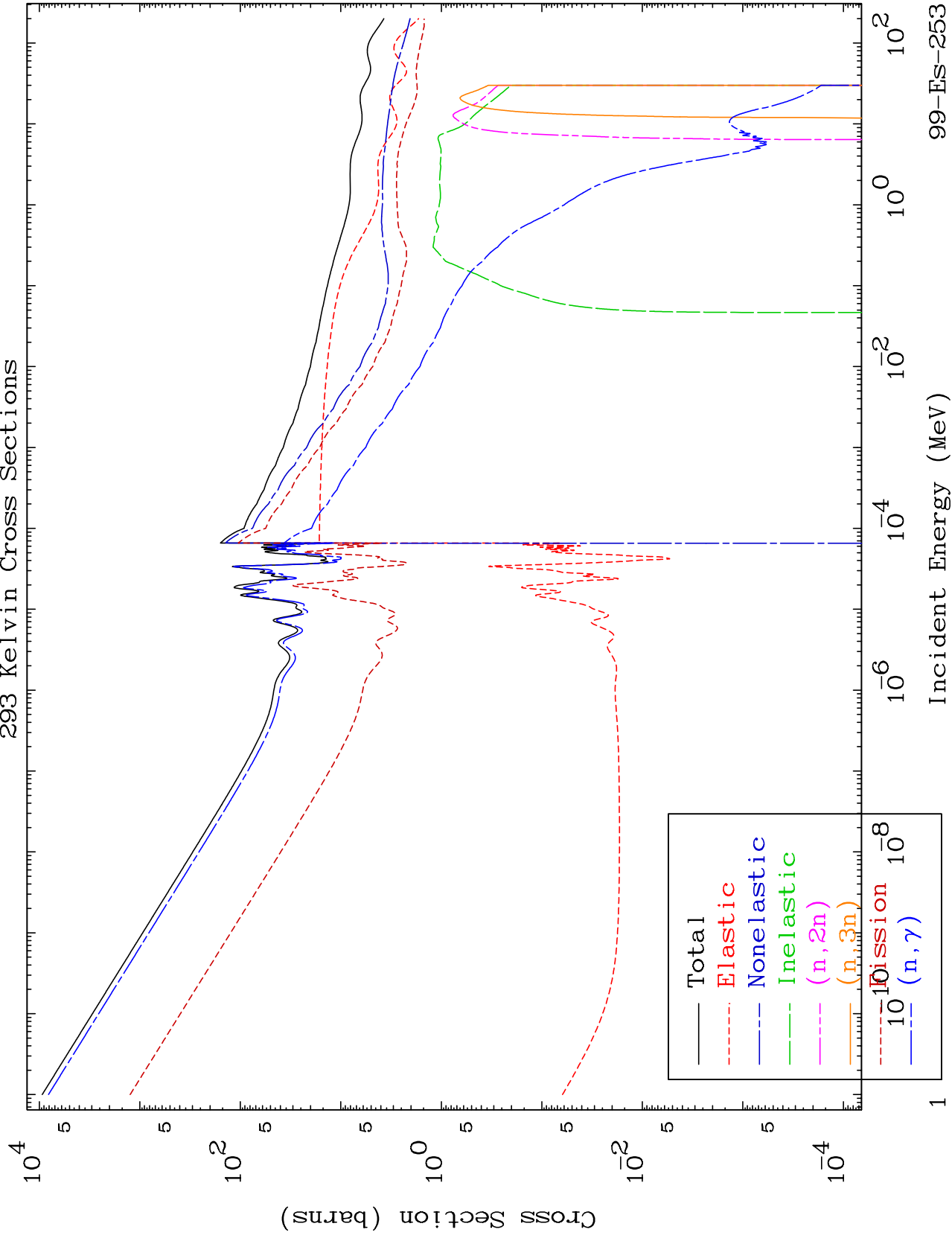
Press Mouse Button to Start

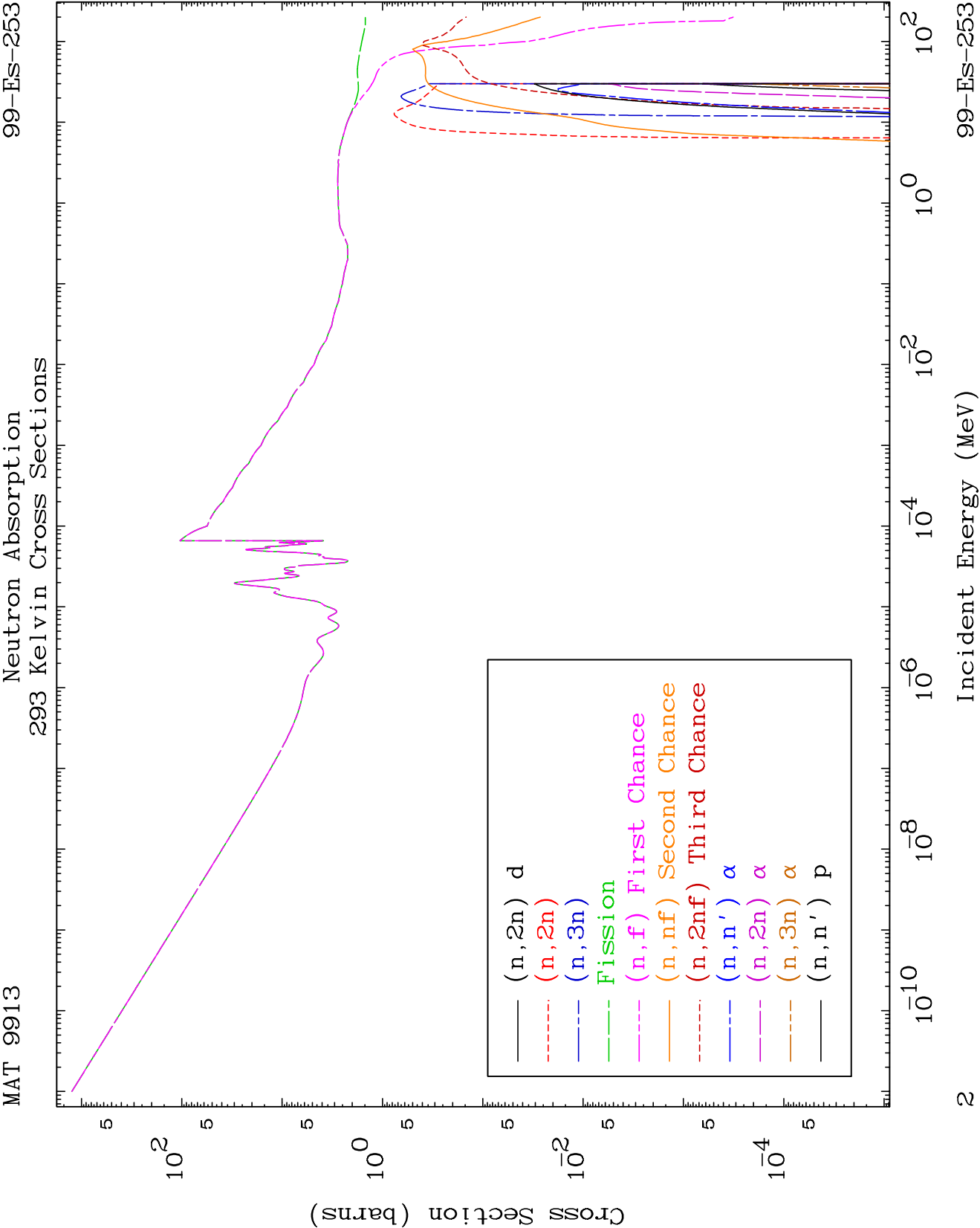
MAT 9913

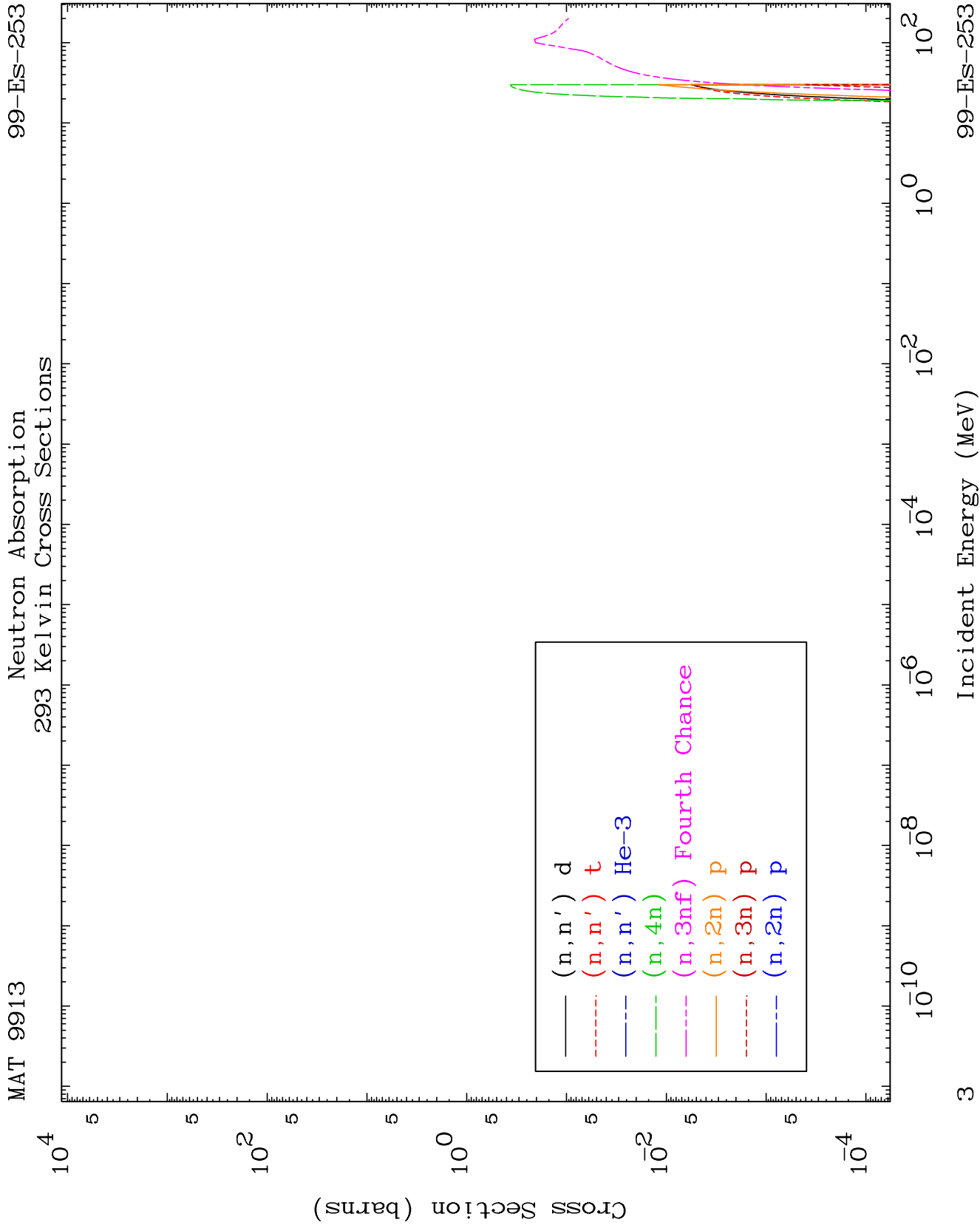
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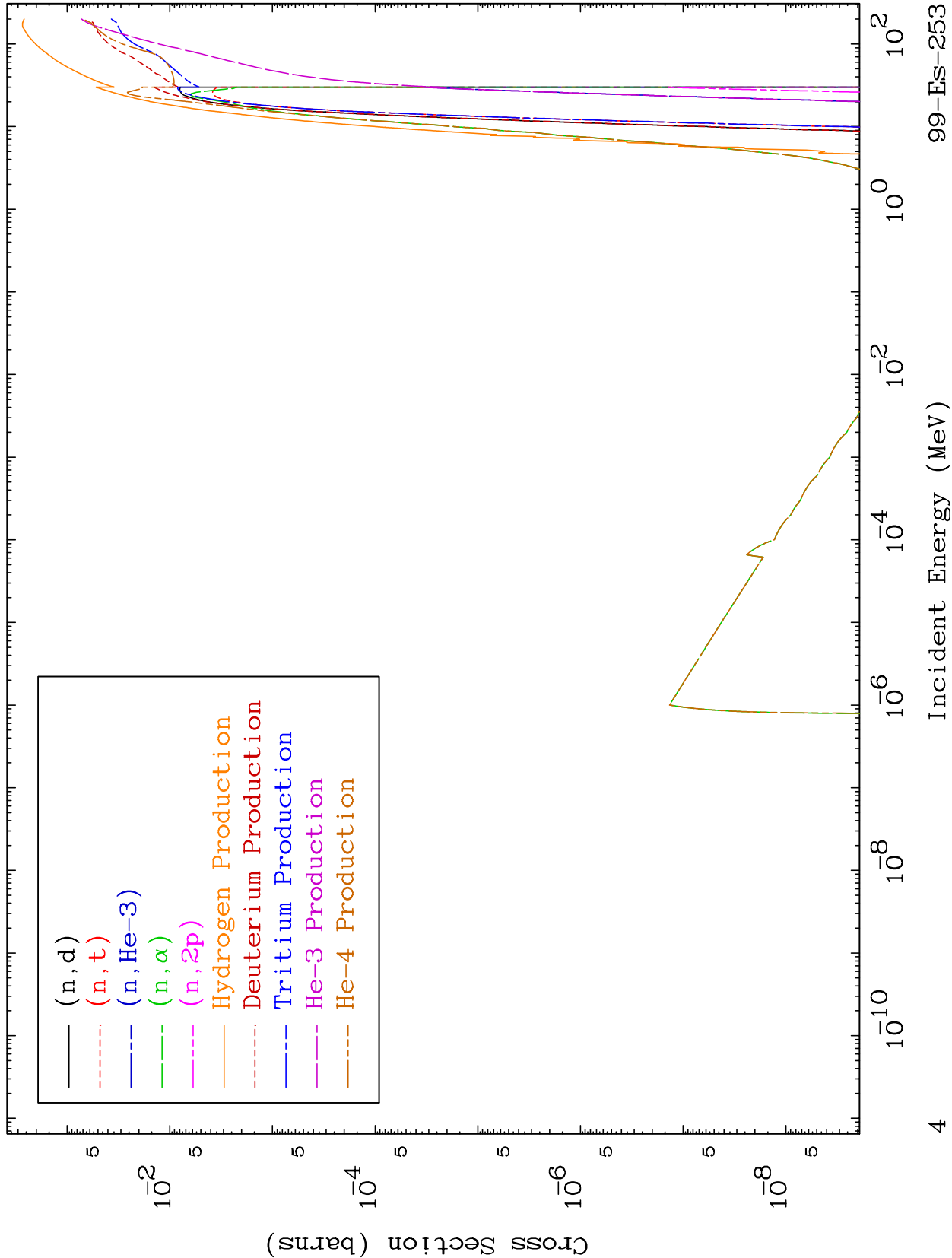
Neutron Major
293 Kelvin Cross Sections

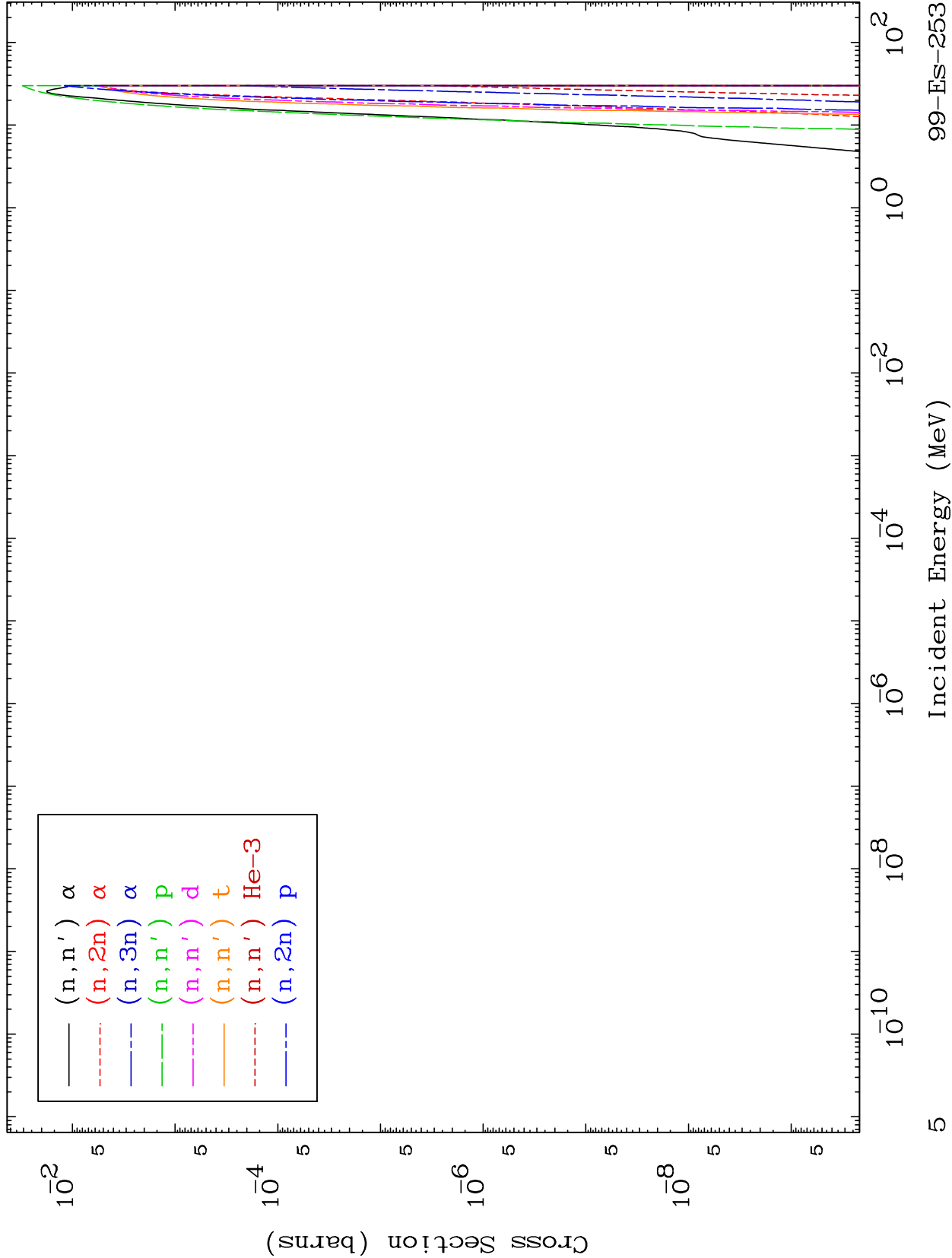
99-ES-253

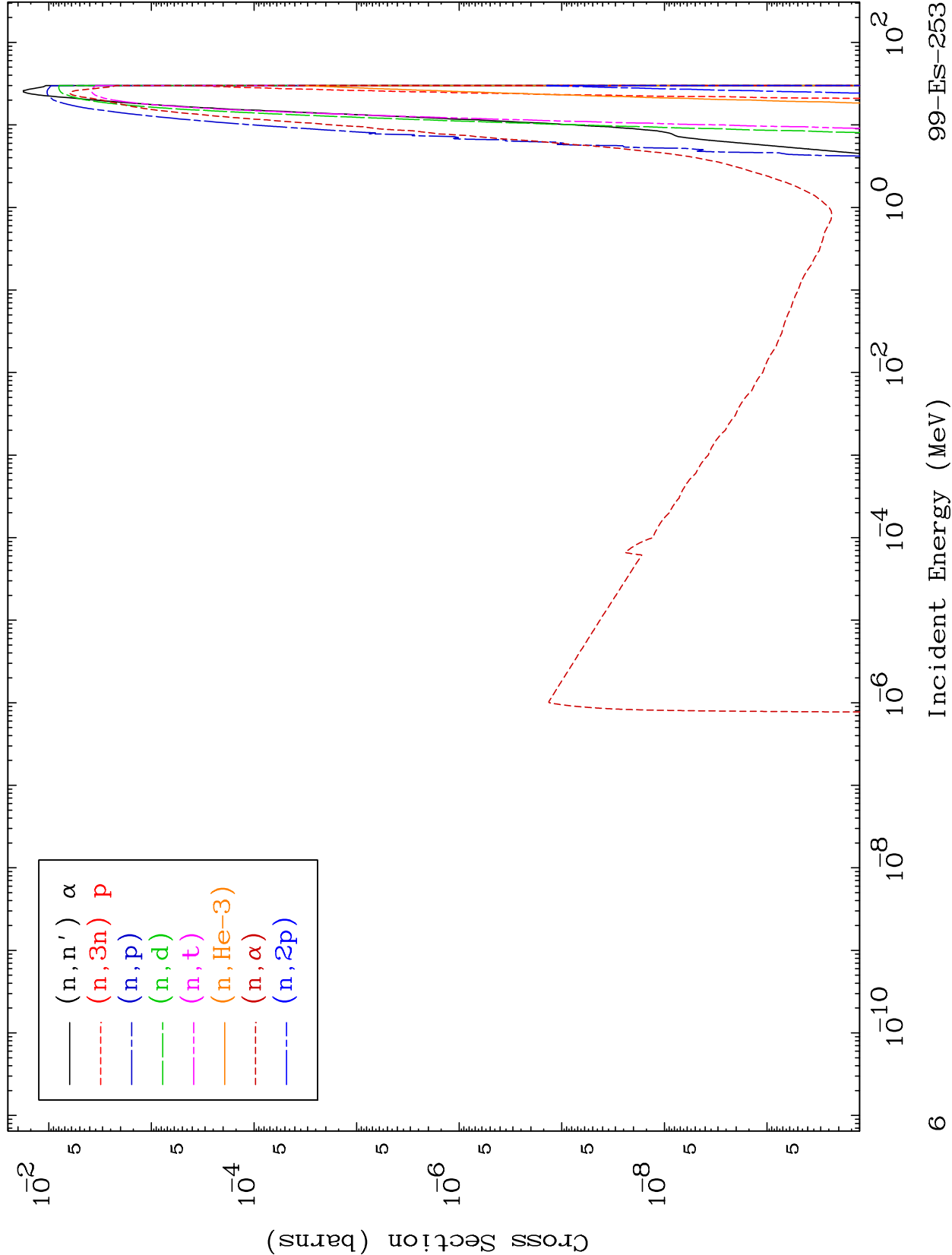


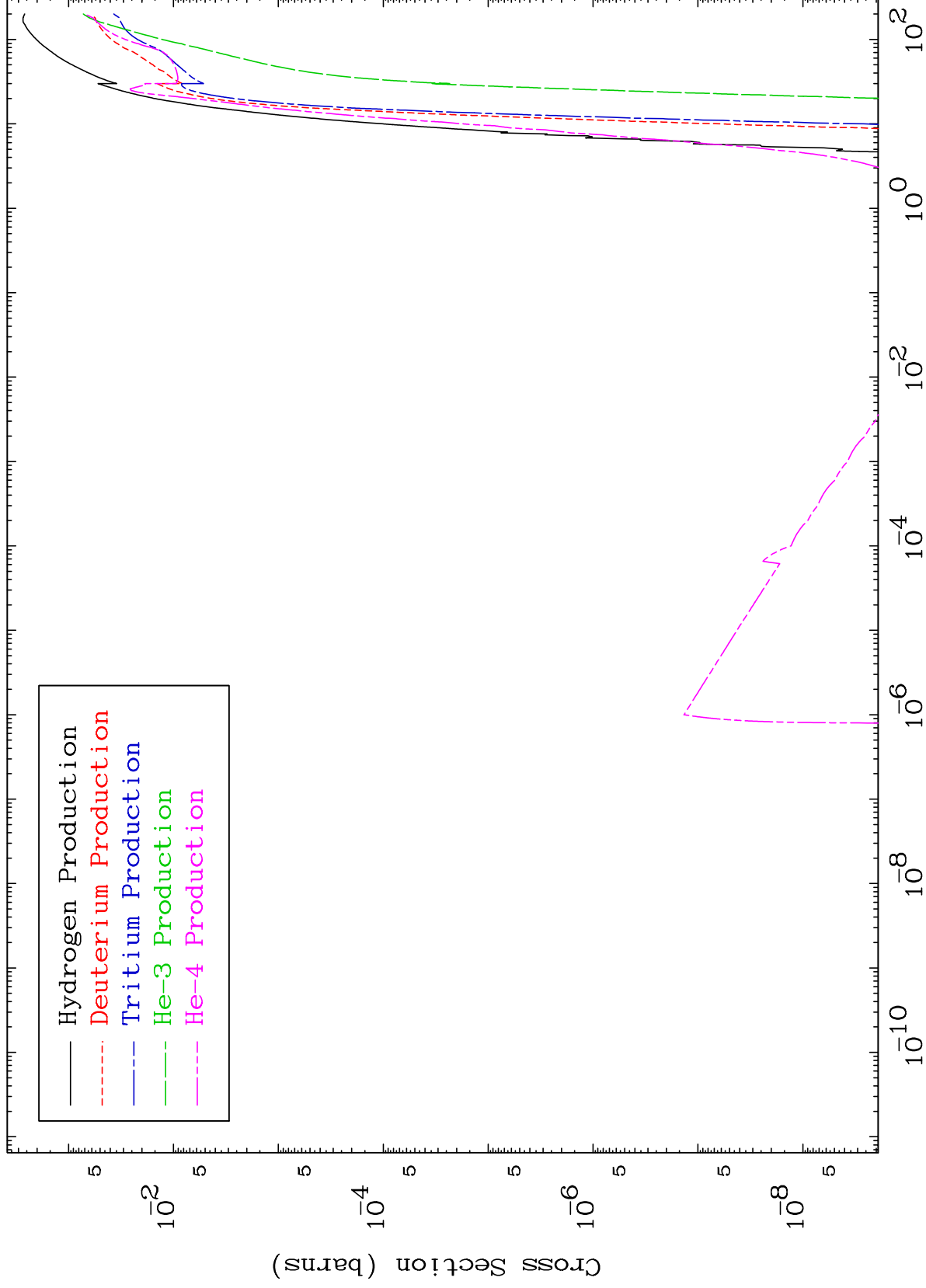


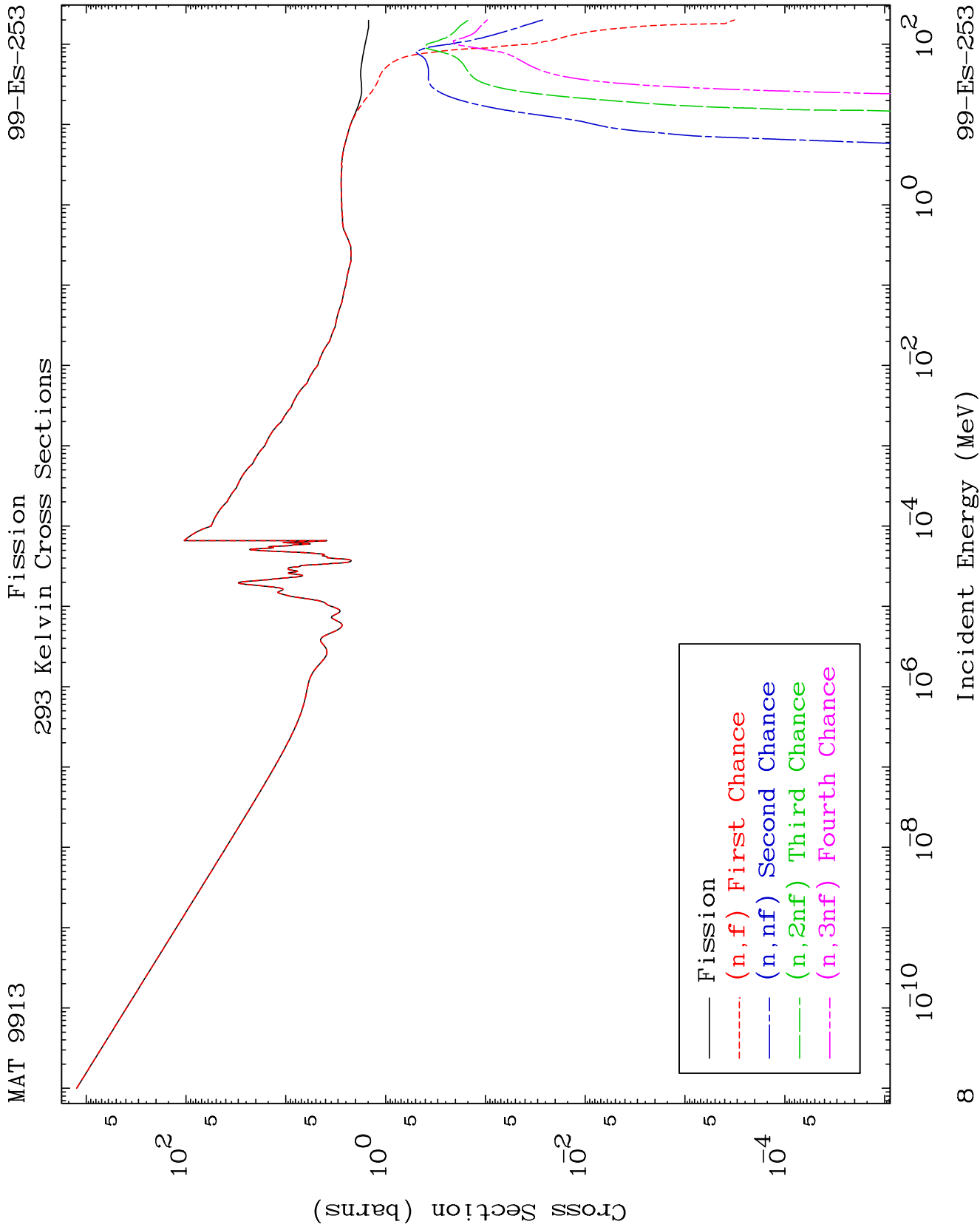


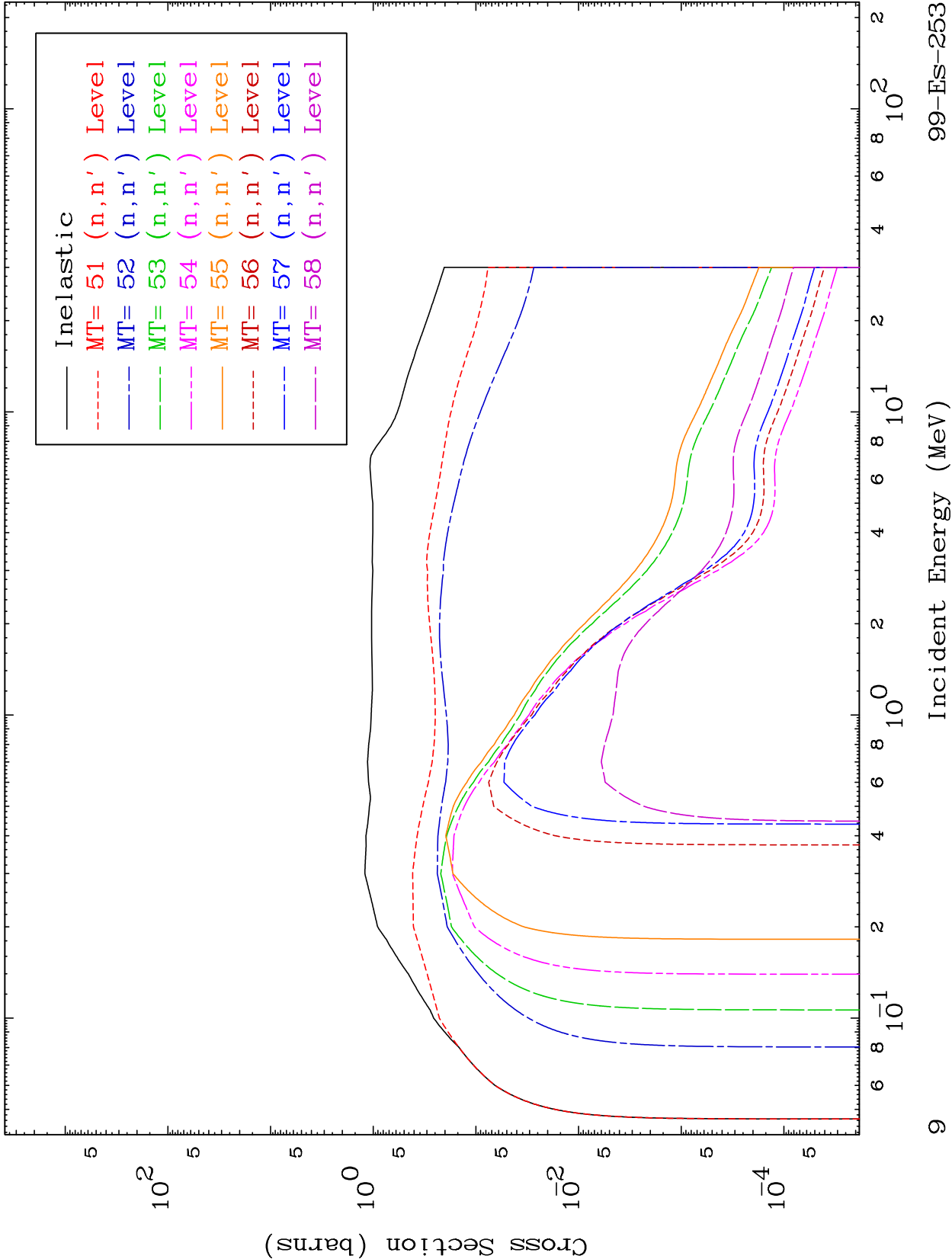


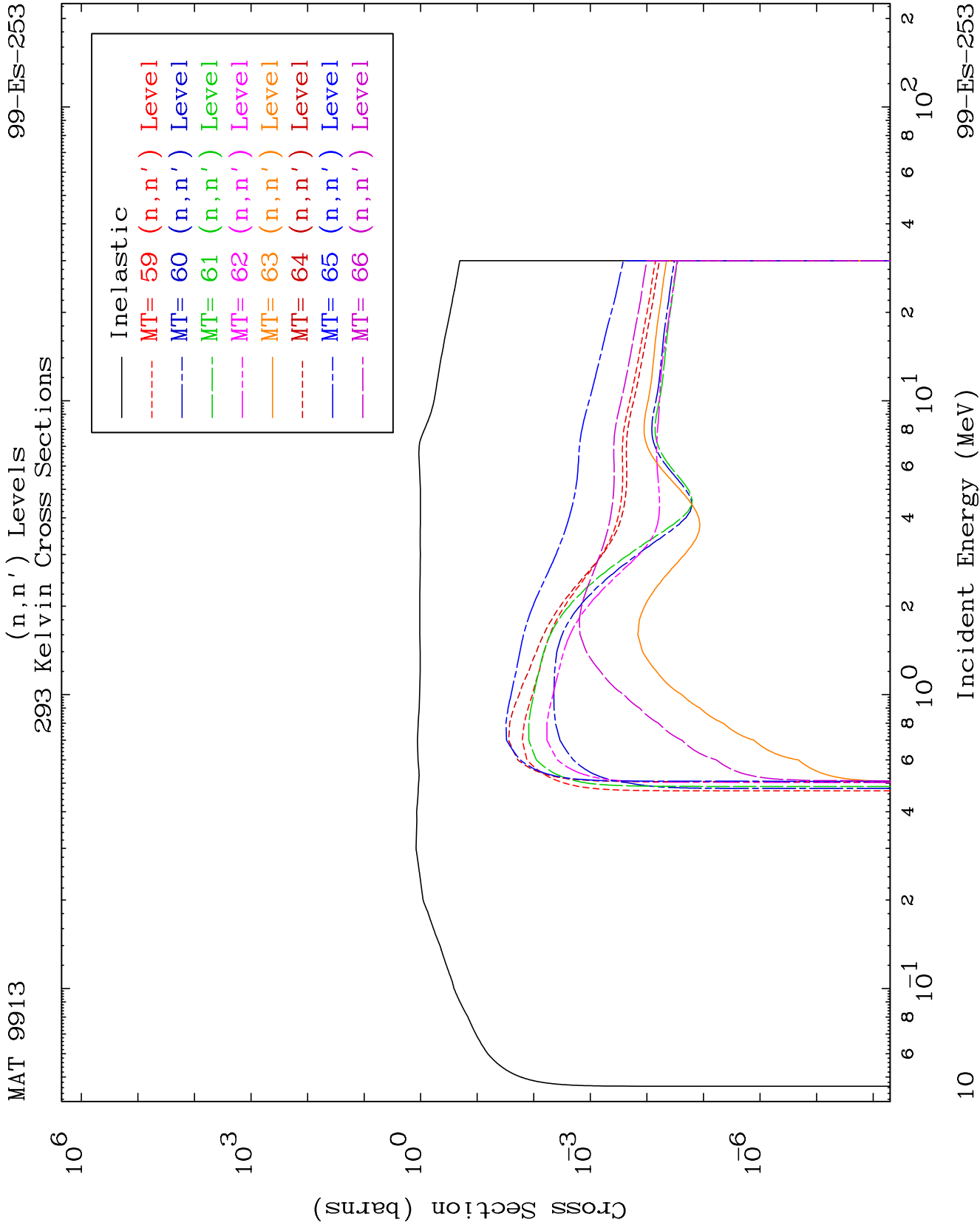


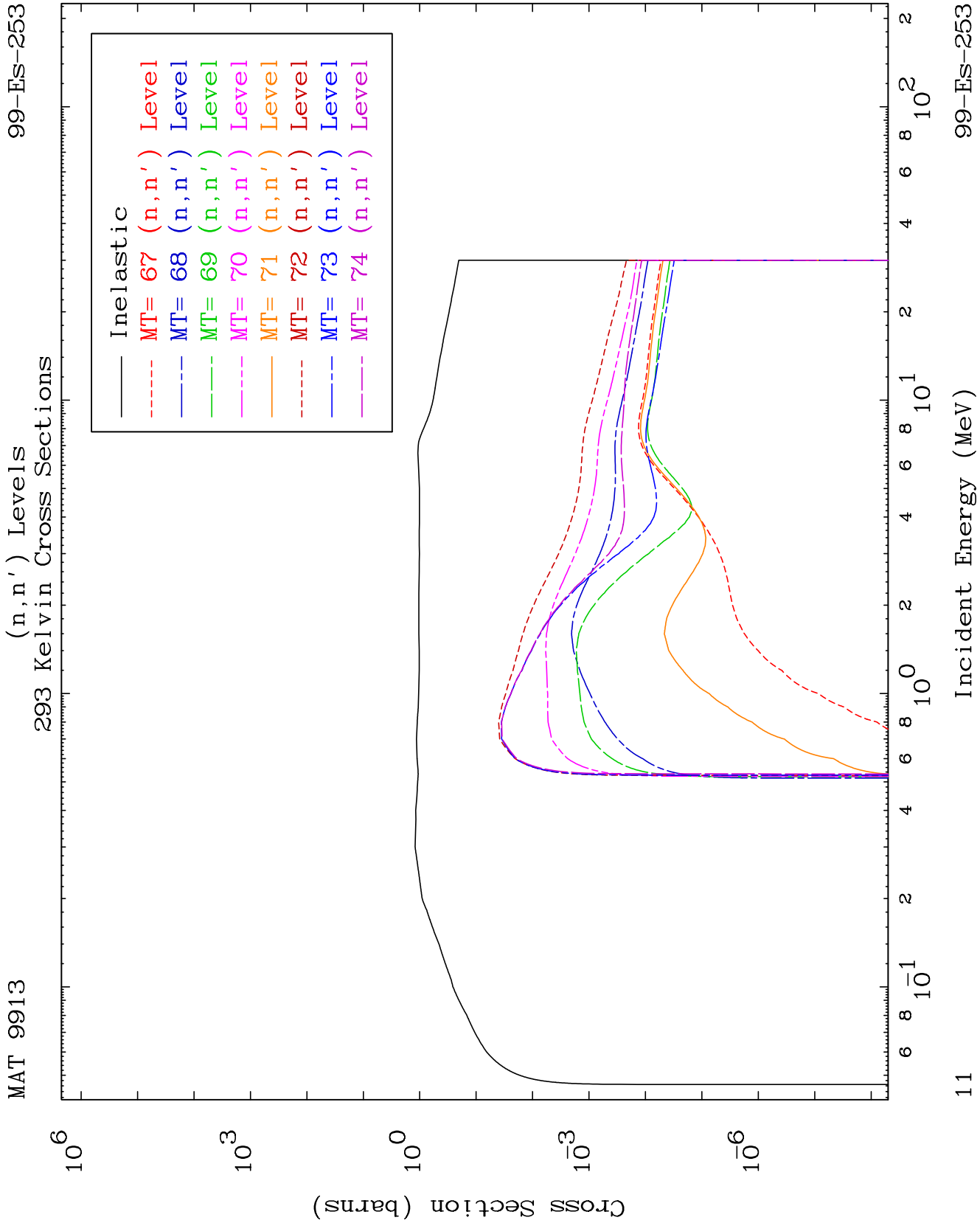


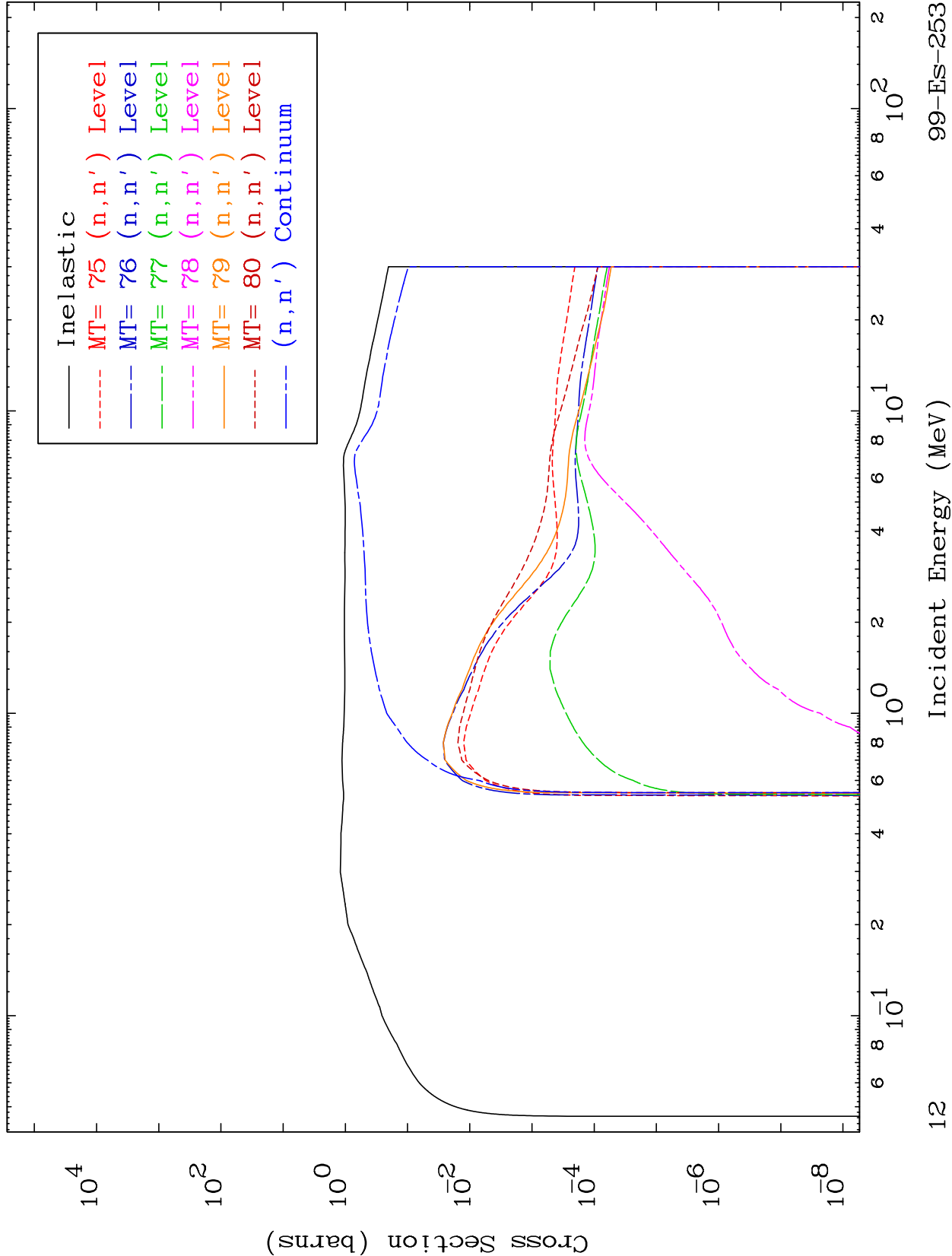


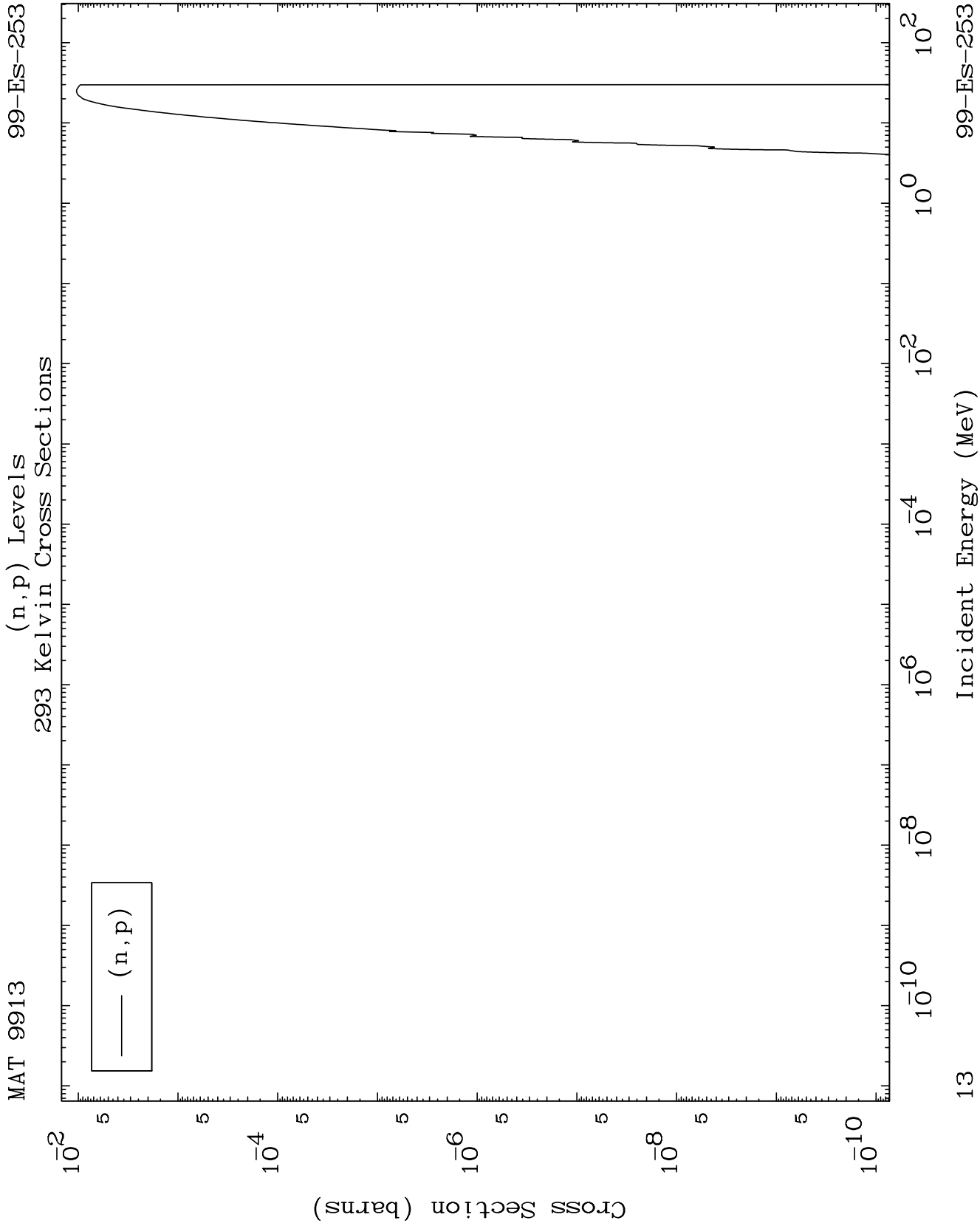


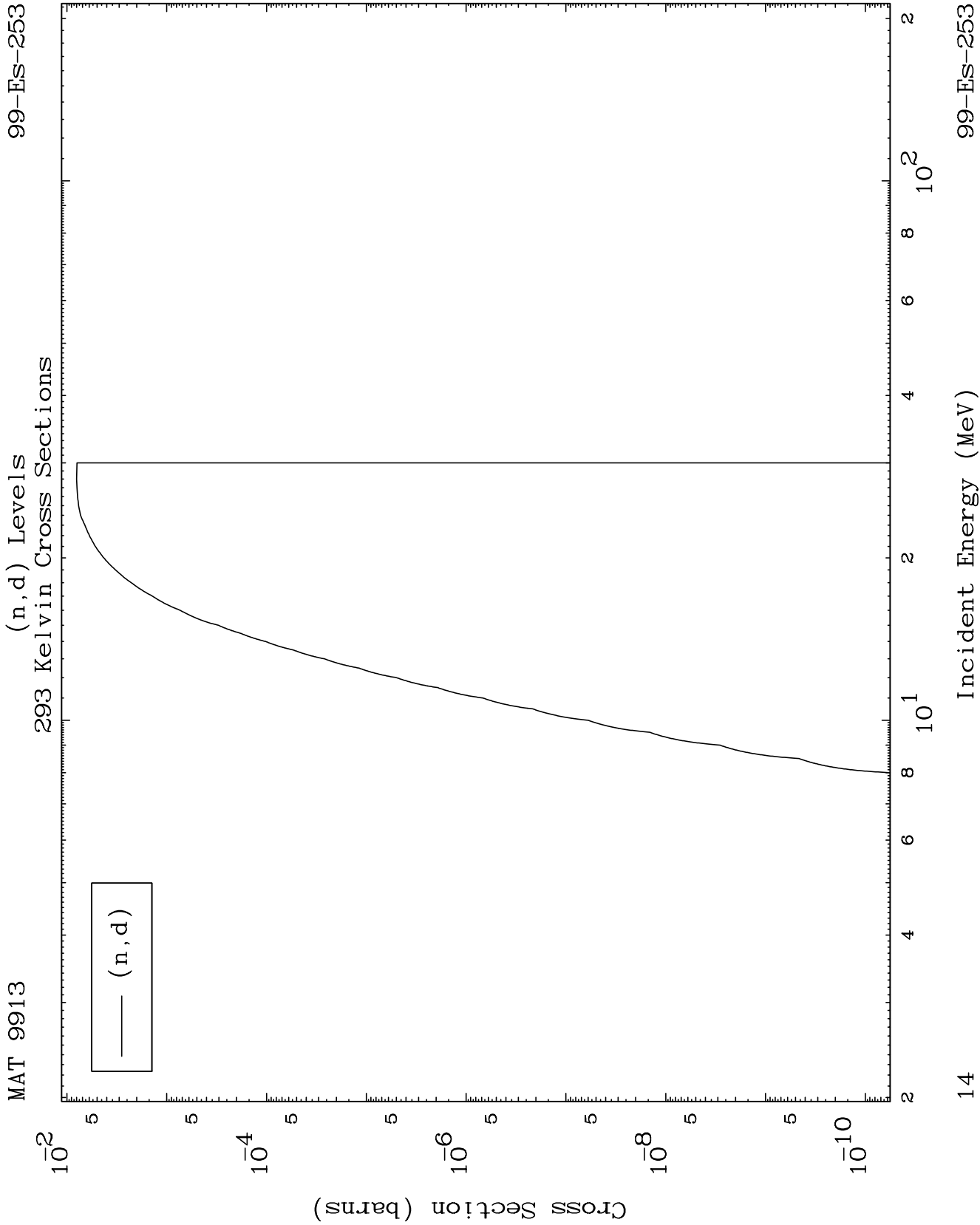


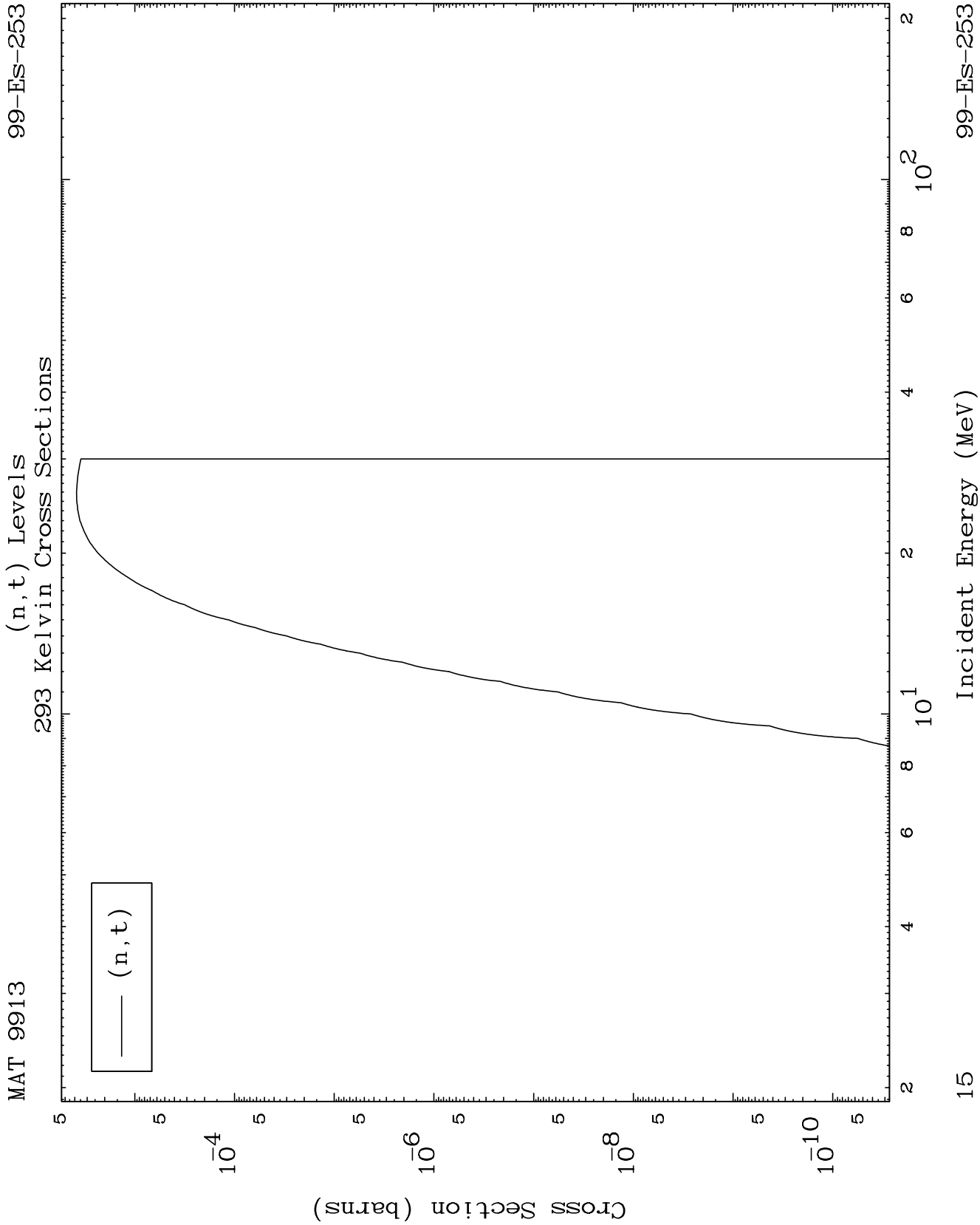








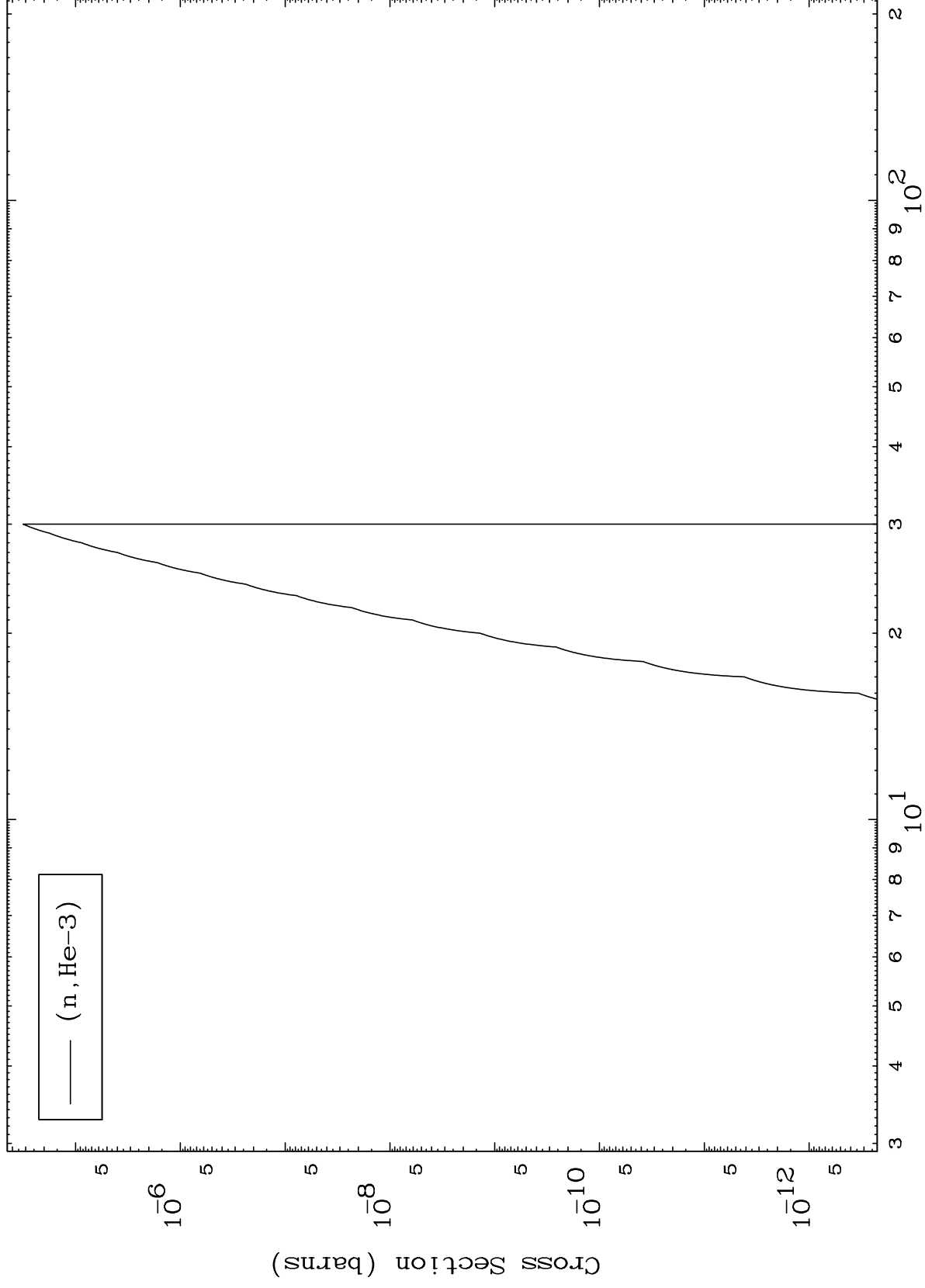




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(n,He3) Levels
293 Kelvin Cross Sections

99-Es-253



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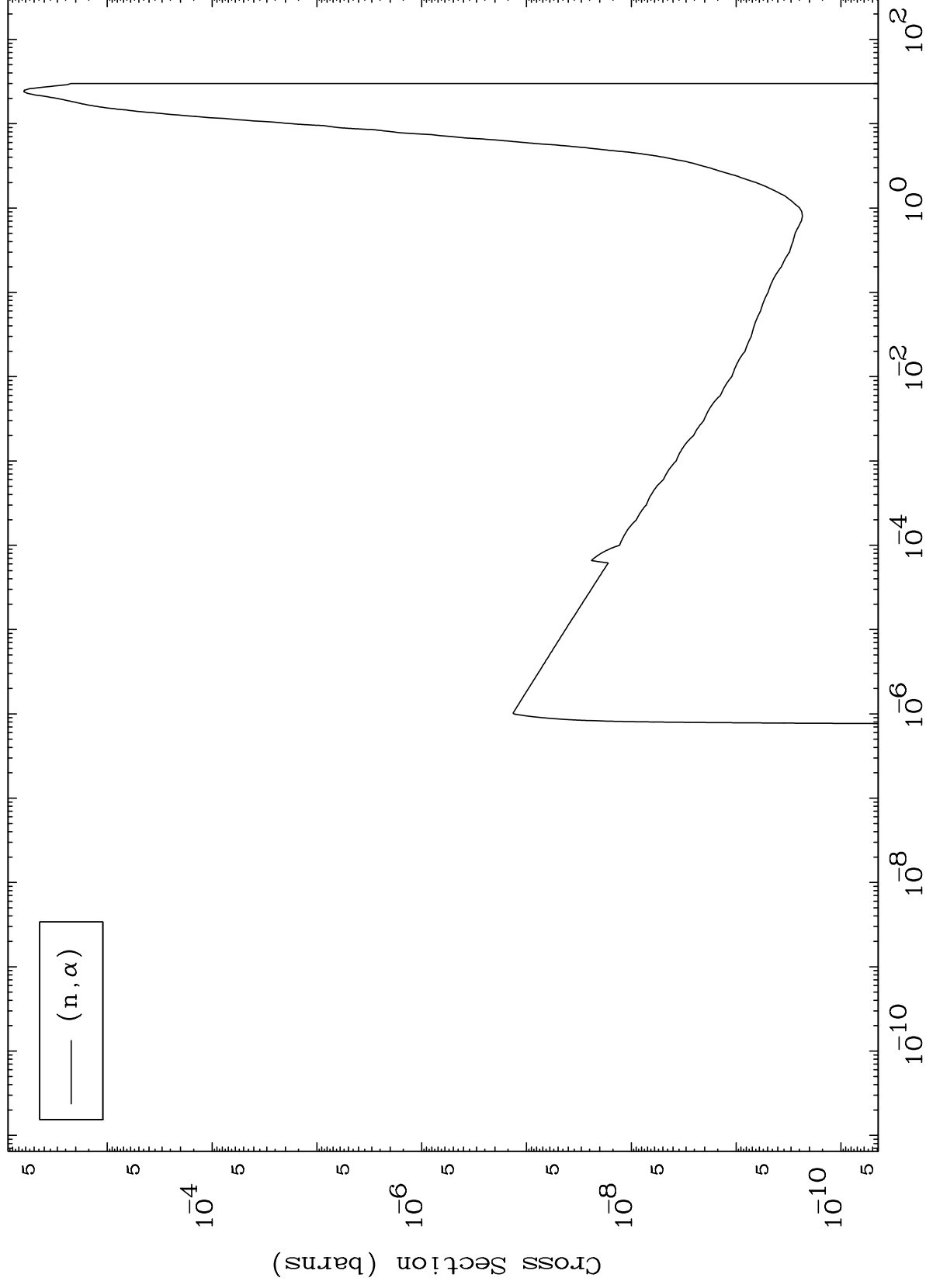
Incident Energy (MeV)

99-Es-253

MAT 9913

(n, α) Levels
293 Kelvin Cross Sections

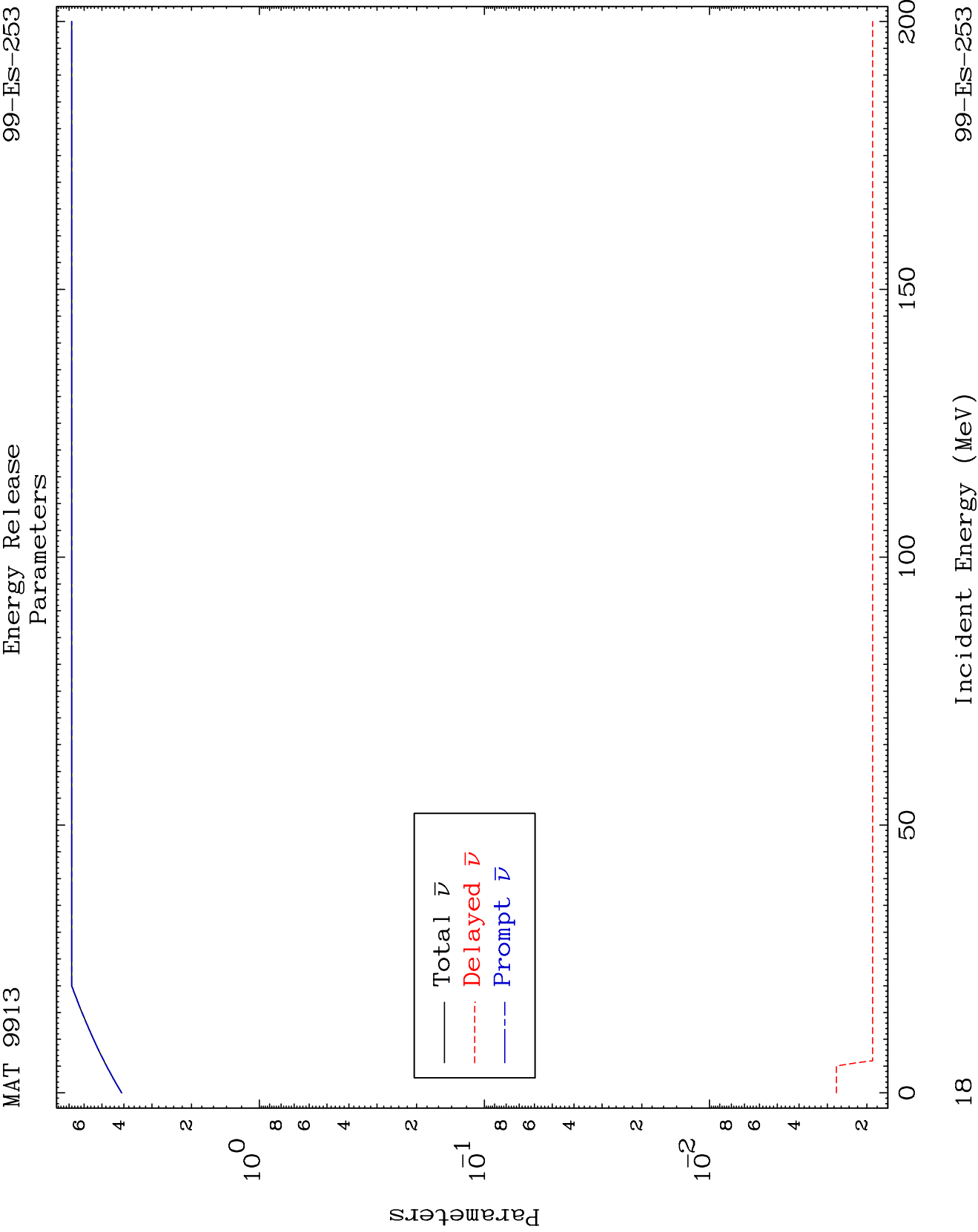
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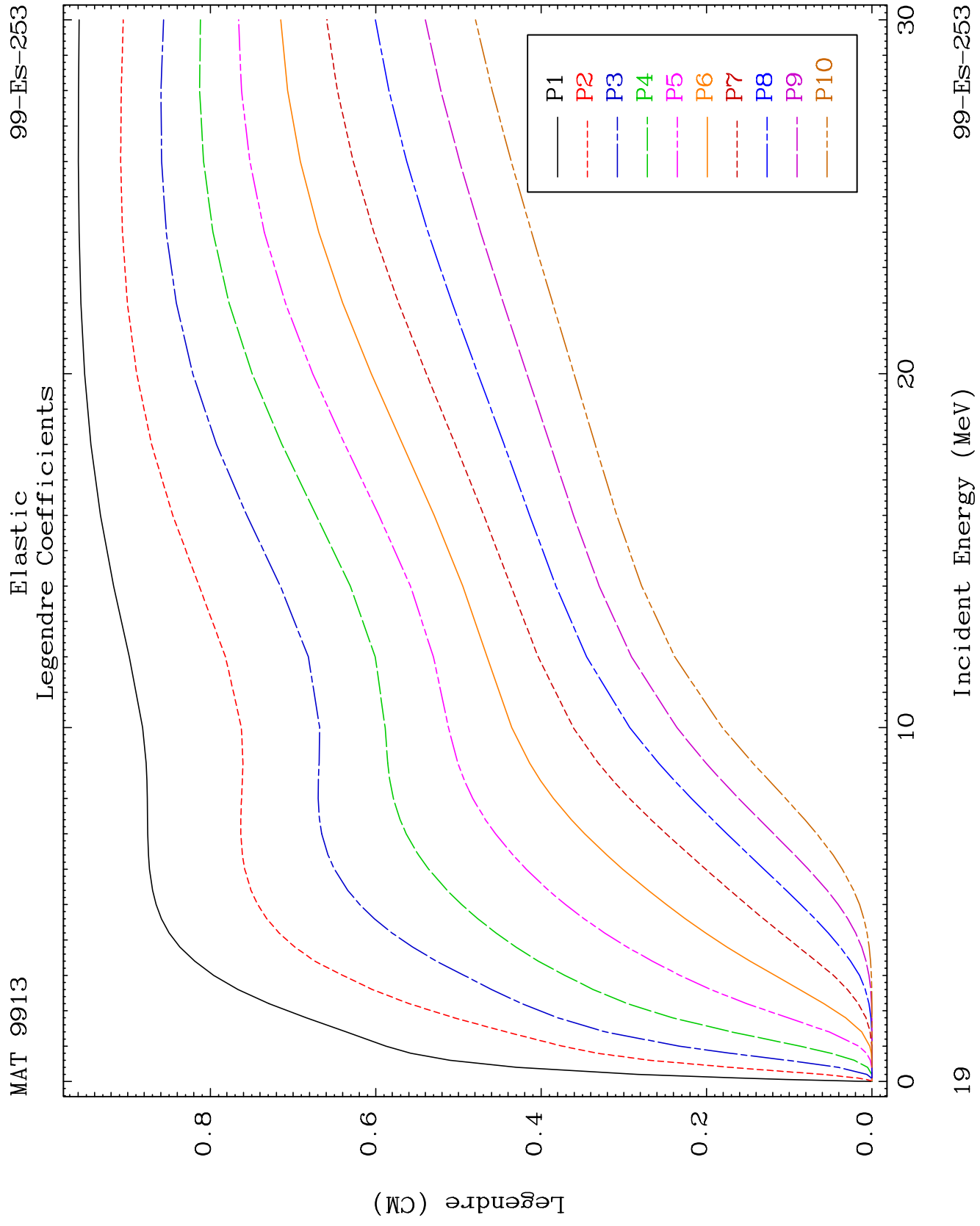


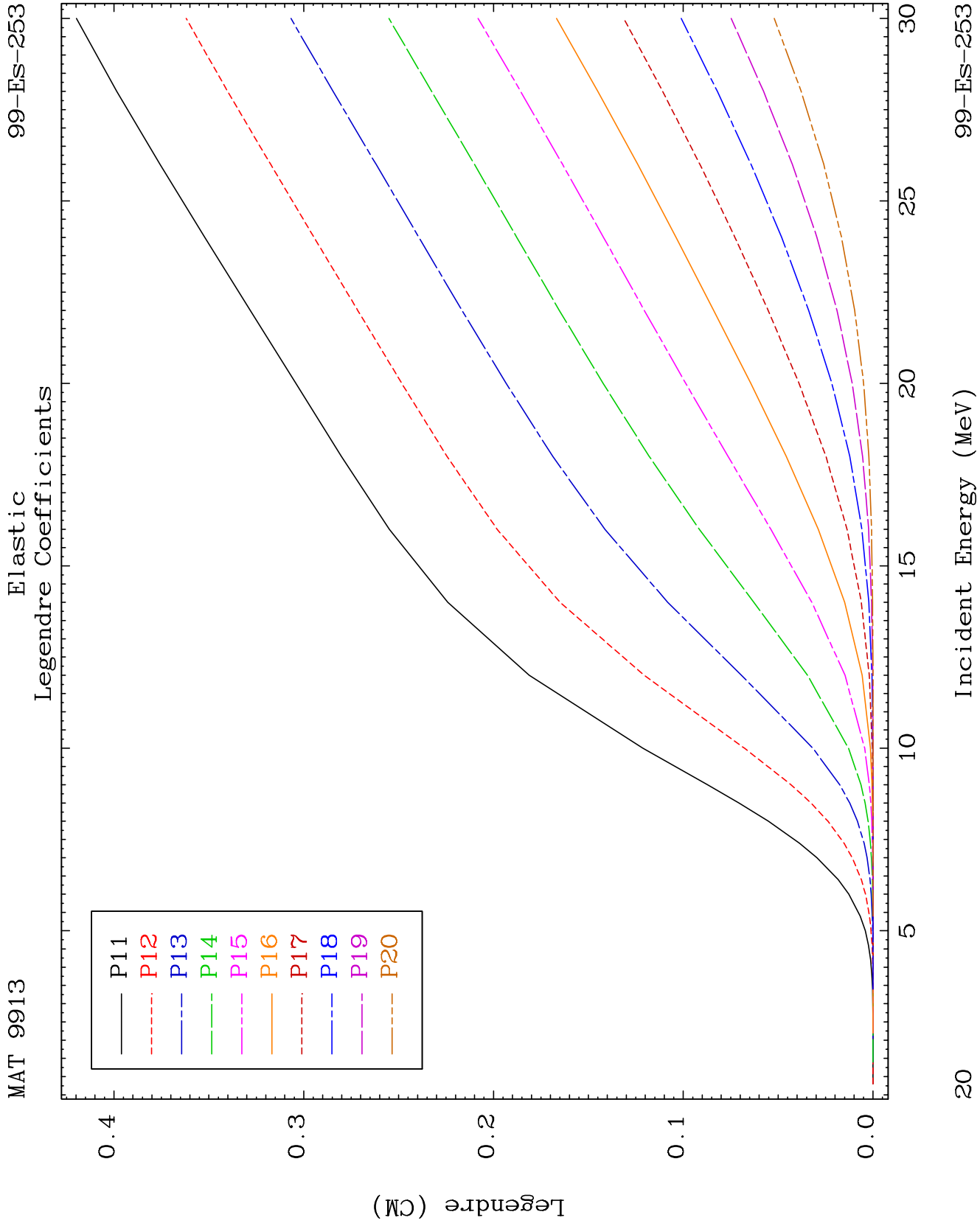
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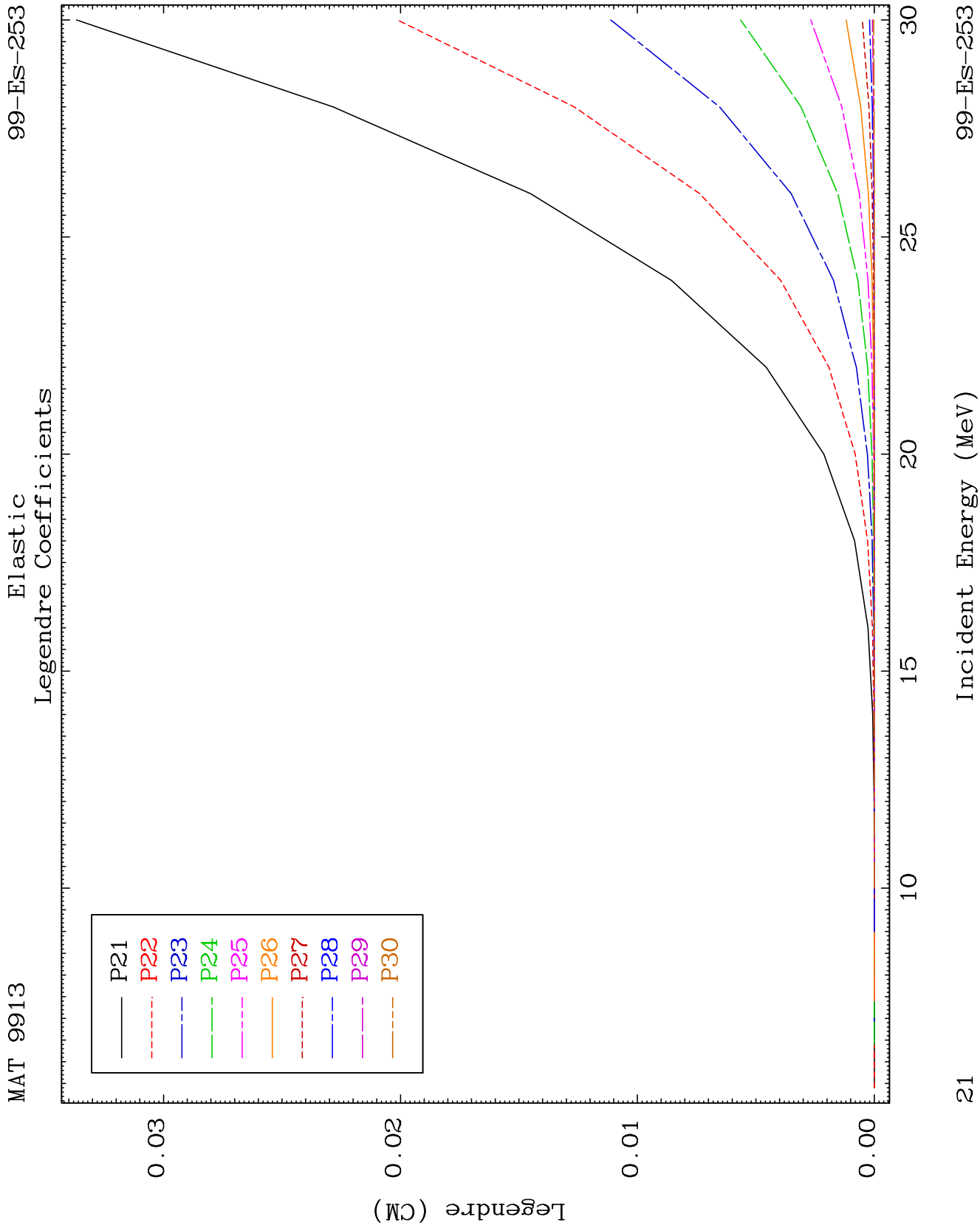
Incident Energy (MeV)

99-Es-253





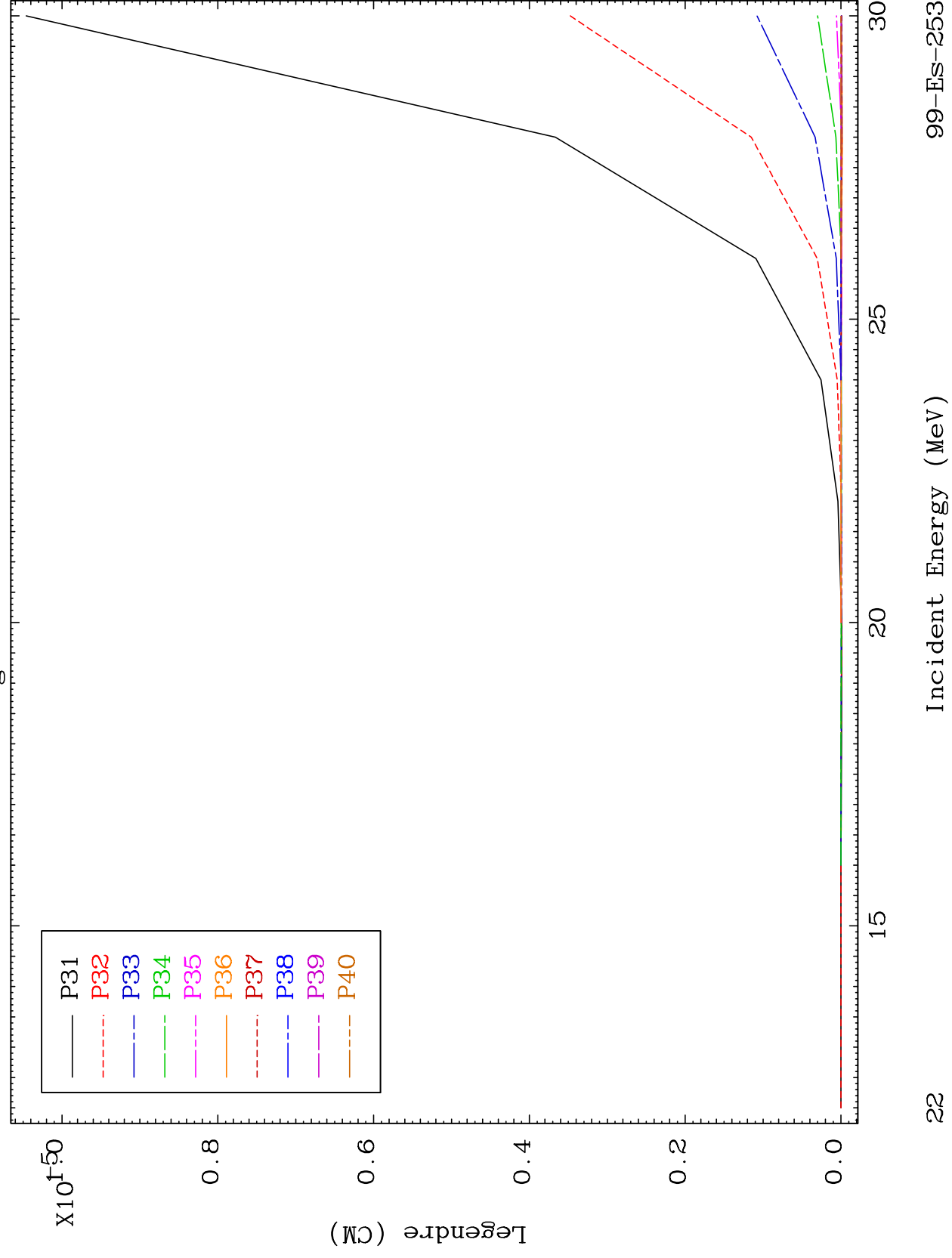


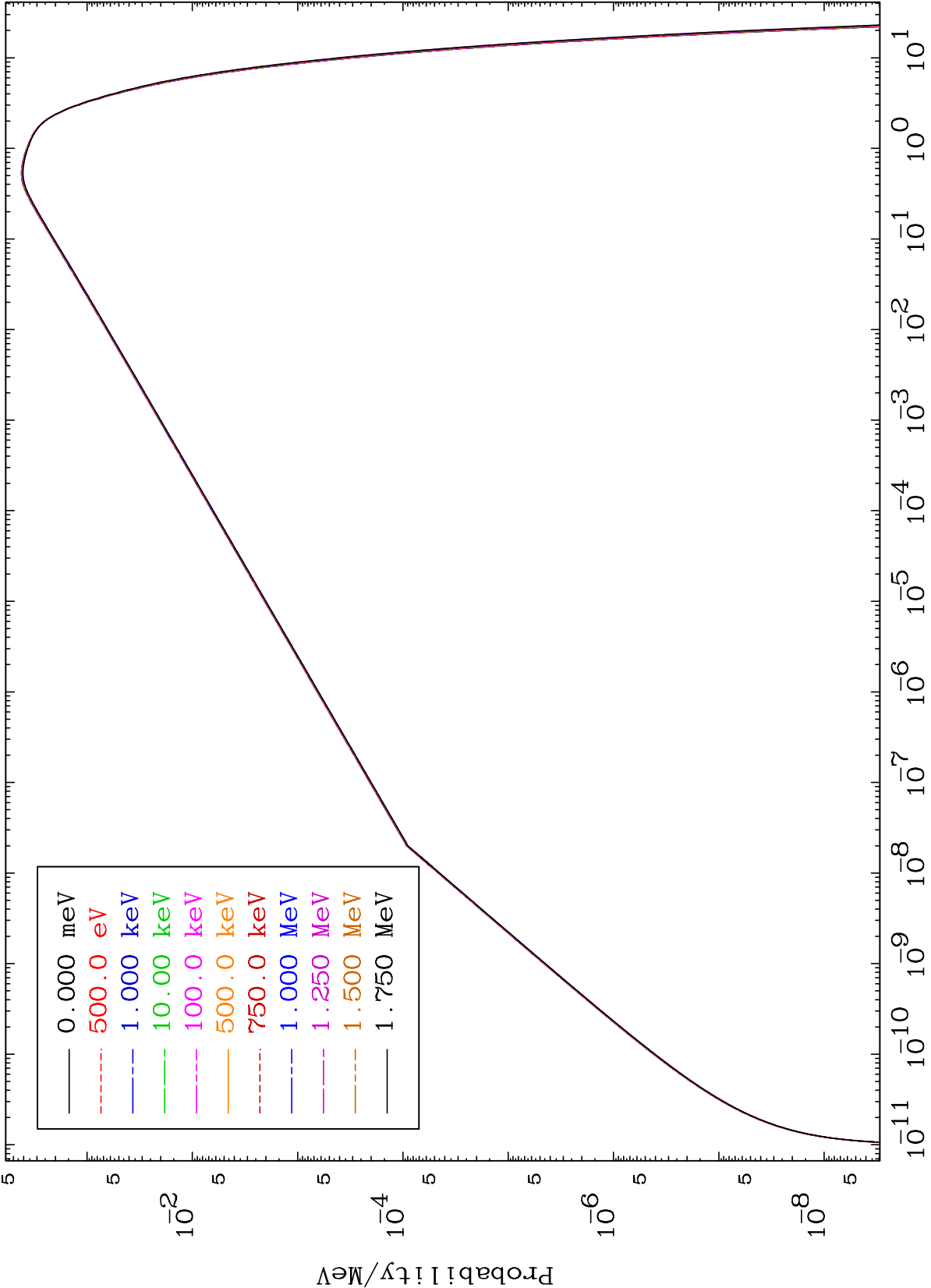


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Elastic Legendre Coefficients

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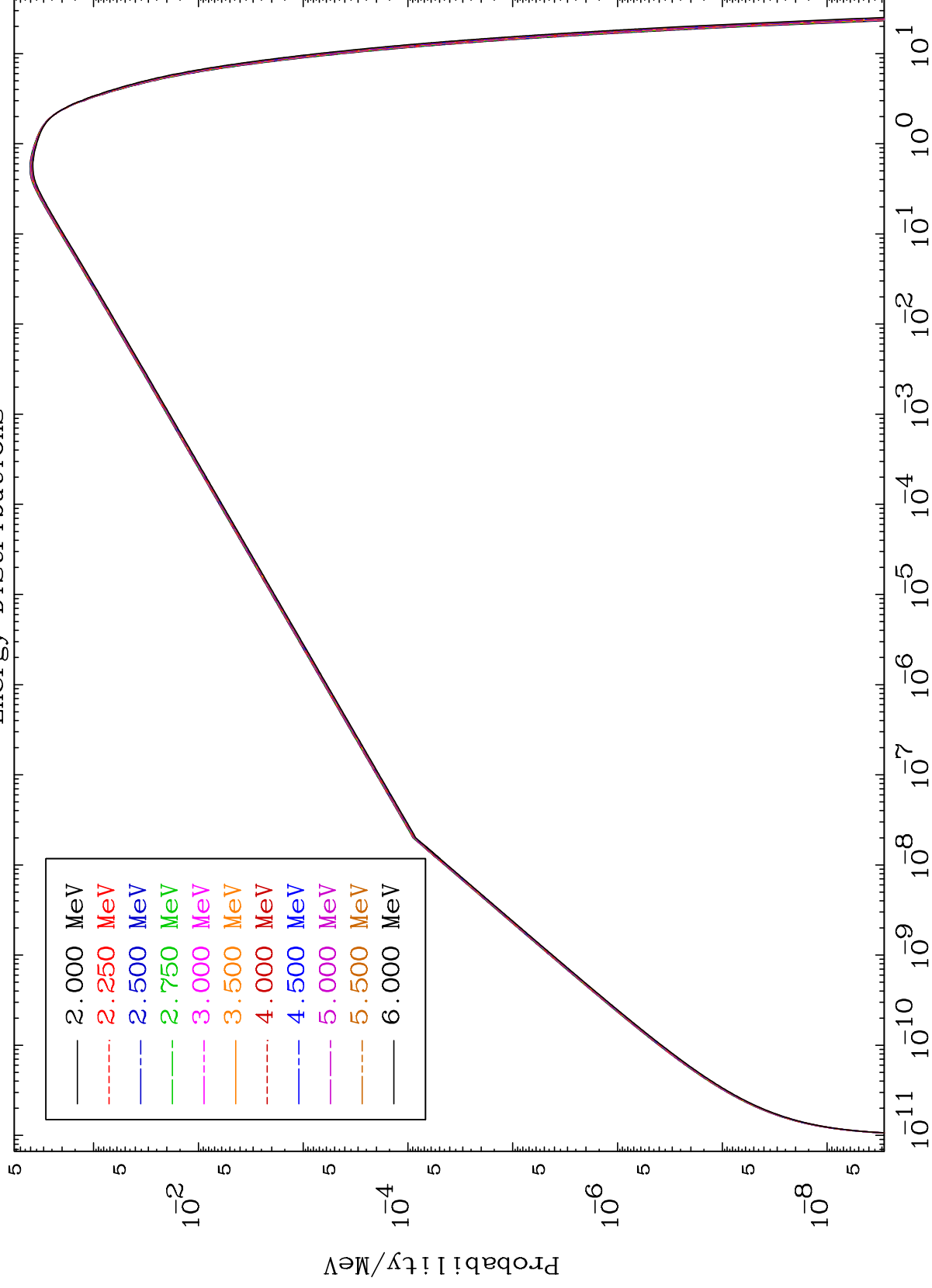




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Fission
Energy Distributions

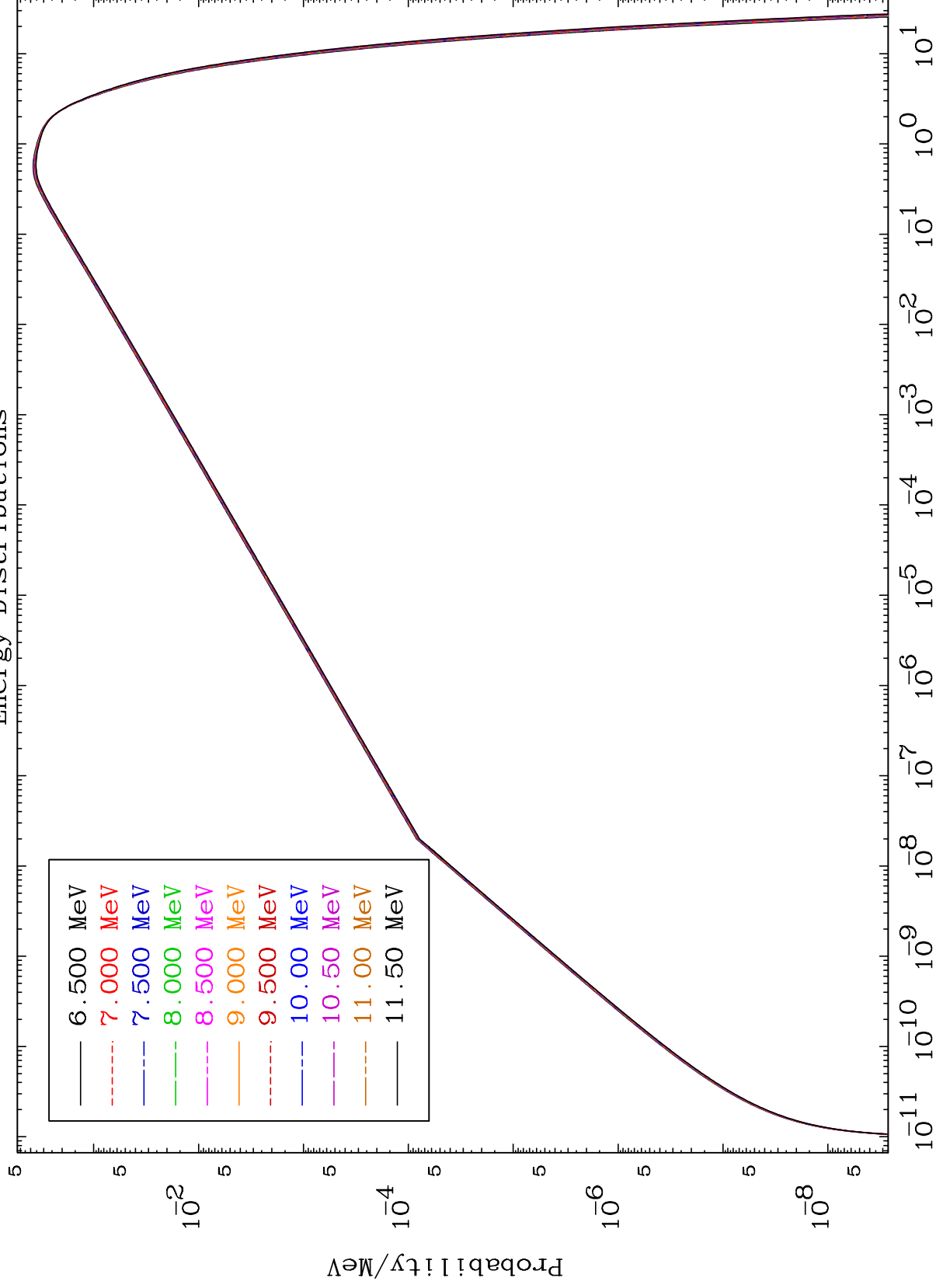
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Fission
Energy Distributions

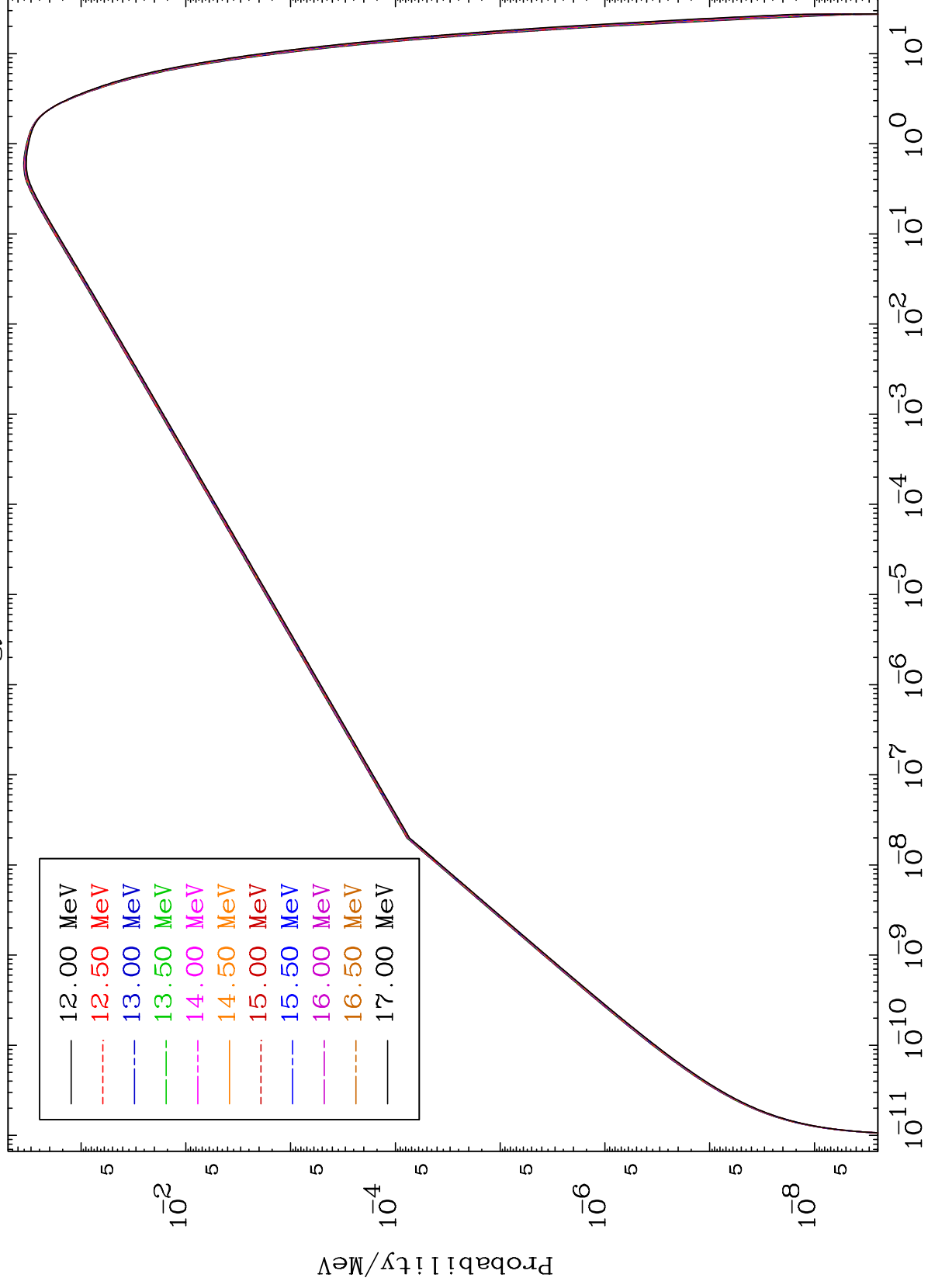
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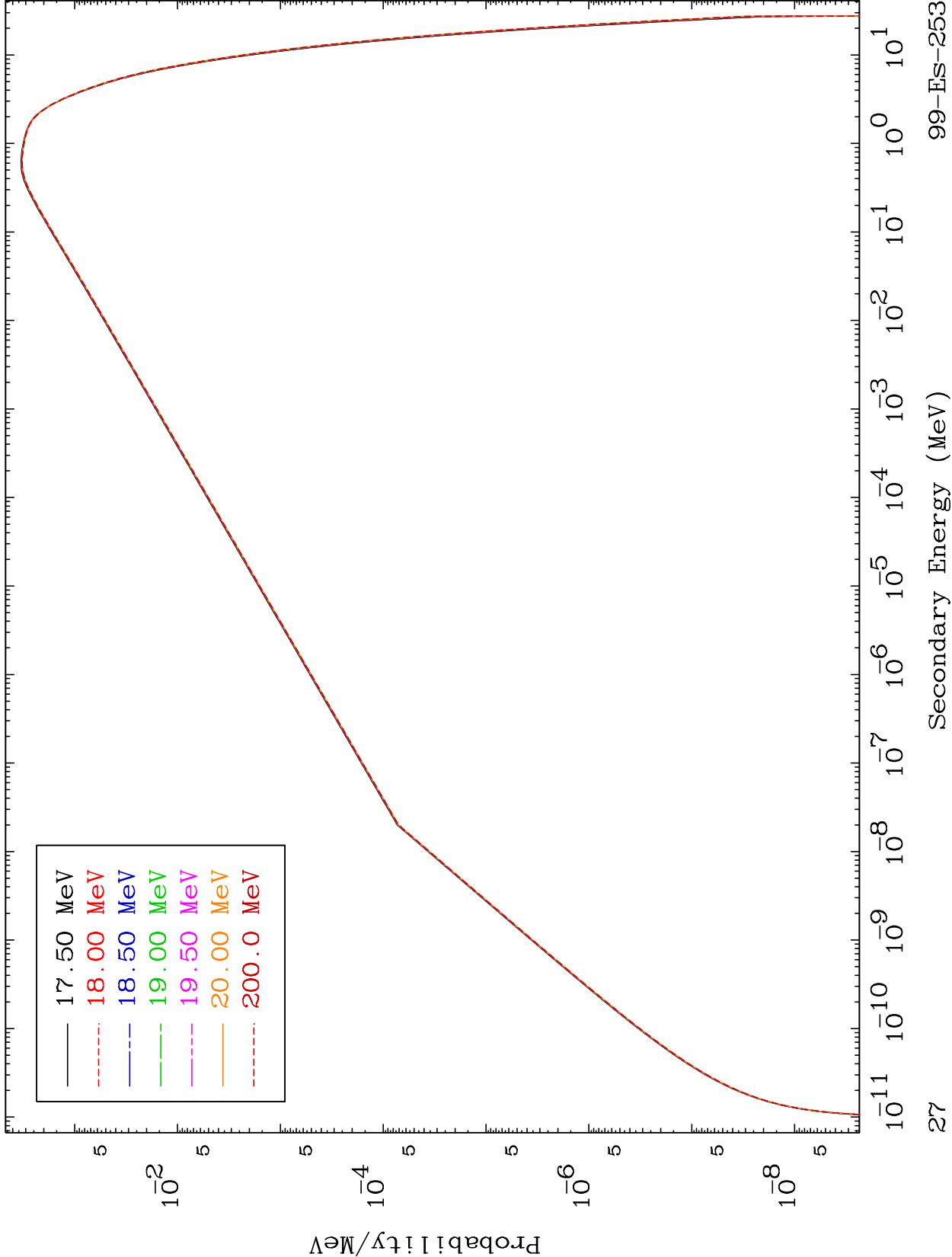


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Secondary Energy (MeV)

99-Es-253



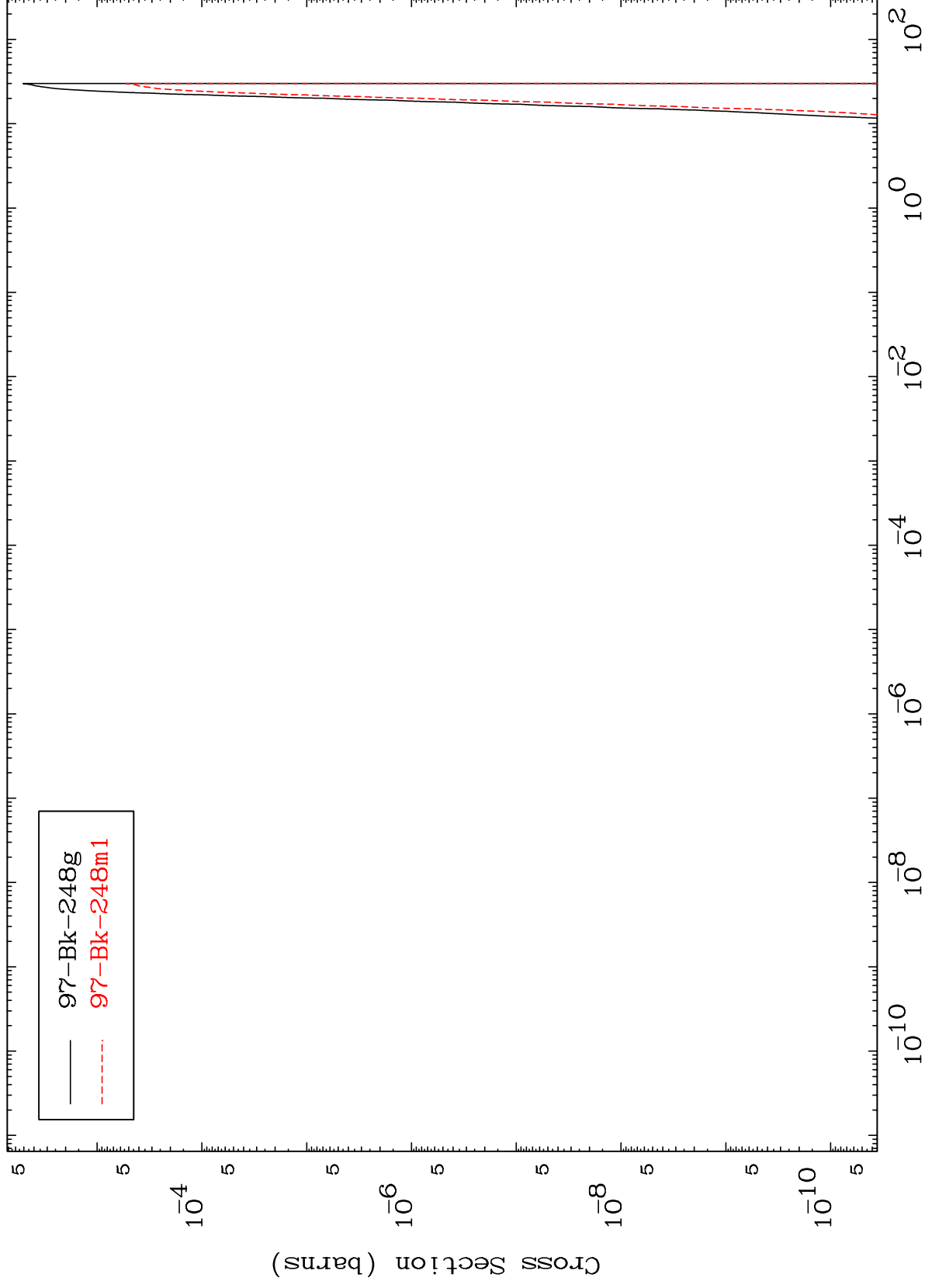


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(n,2n) α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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Radionuclide Production Cross Section

