

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

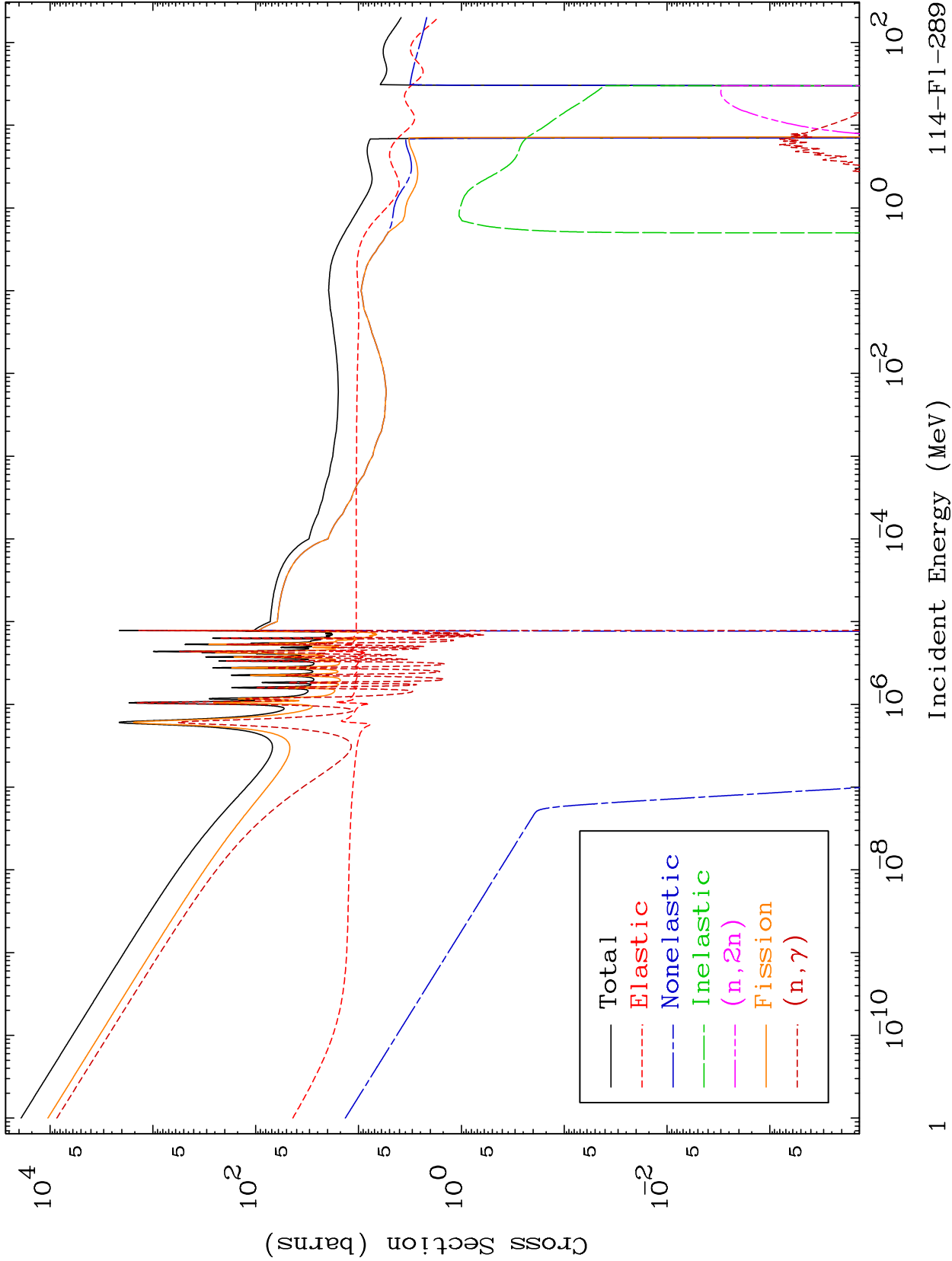
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 1489

Neutron Major  
293 Kelvin Cross Sections

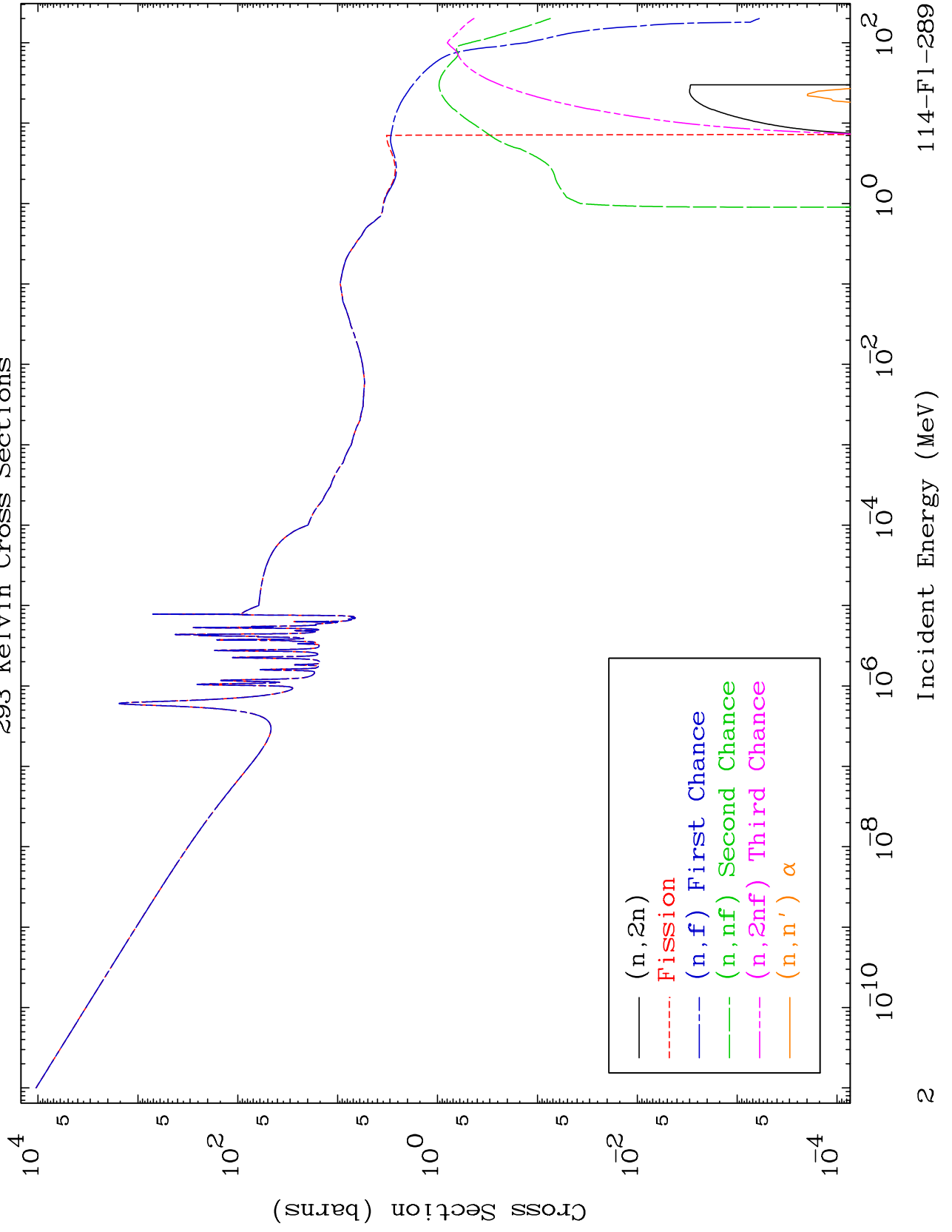
114-F1-289



MAT 1489

Neutron Absorption  
293 Kelvin Cross Sections

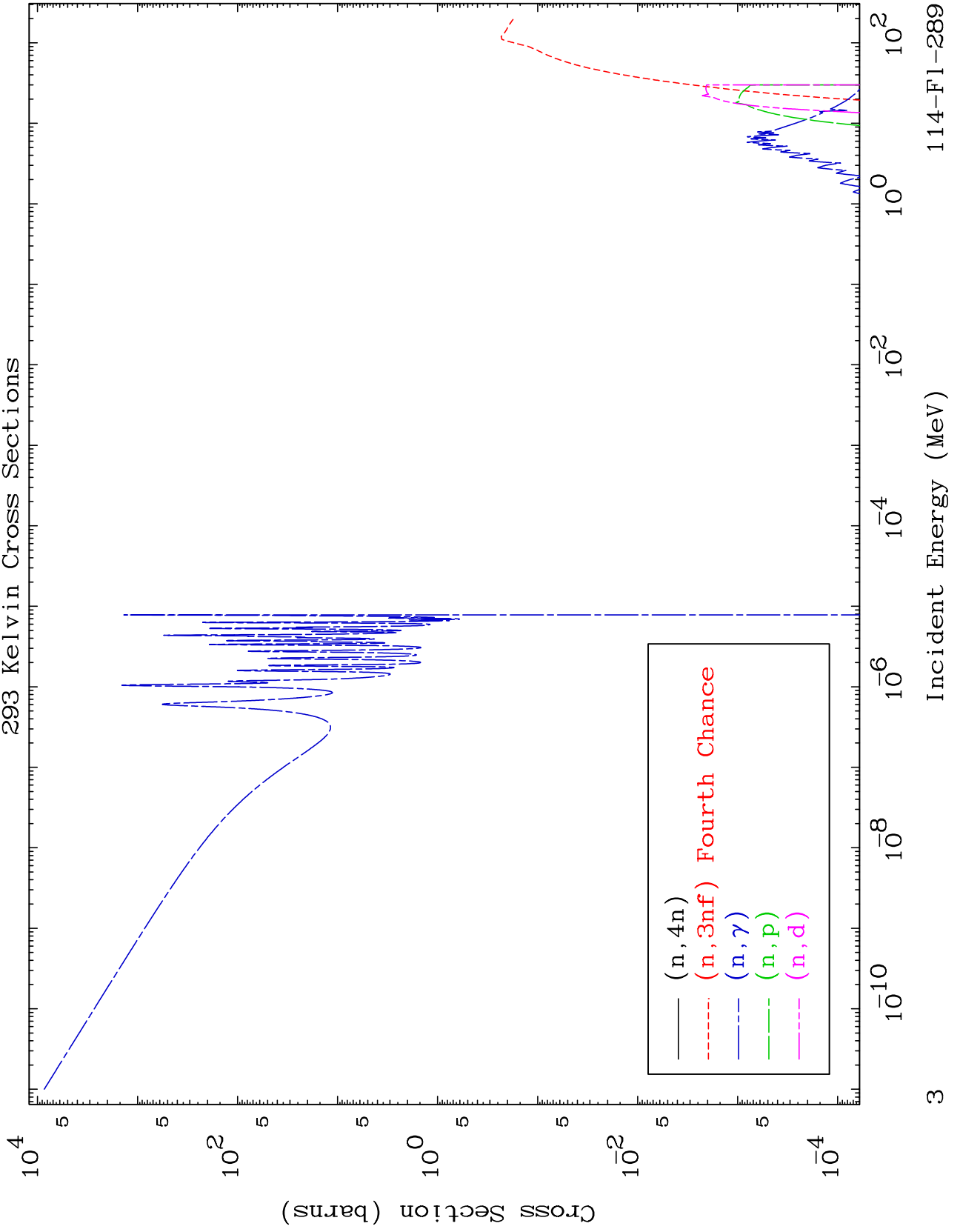
114-F1-289

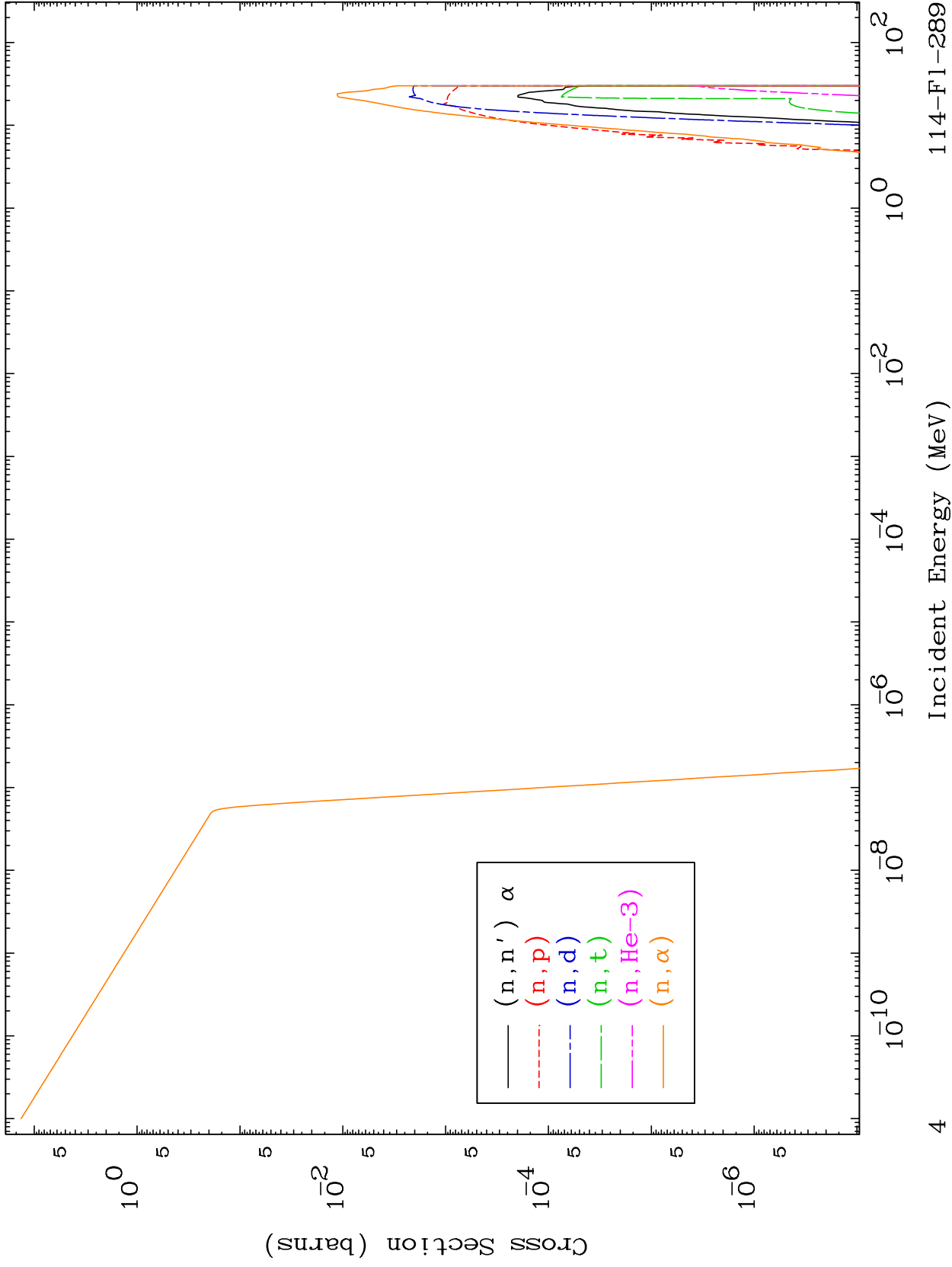


MAT 1489

Neutron Absorption  
293 Kelvin Cross Sections

114-FI-289

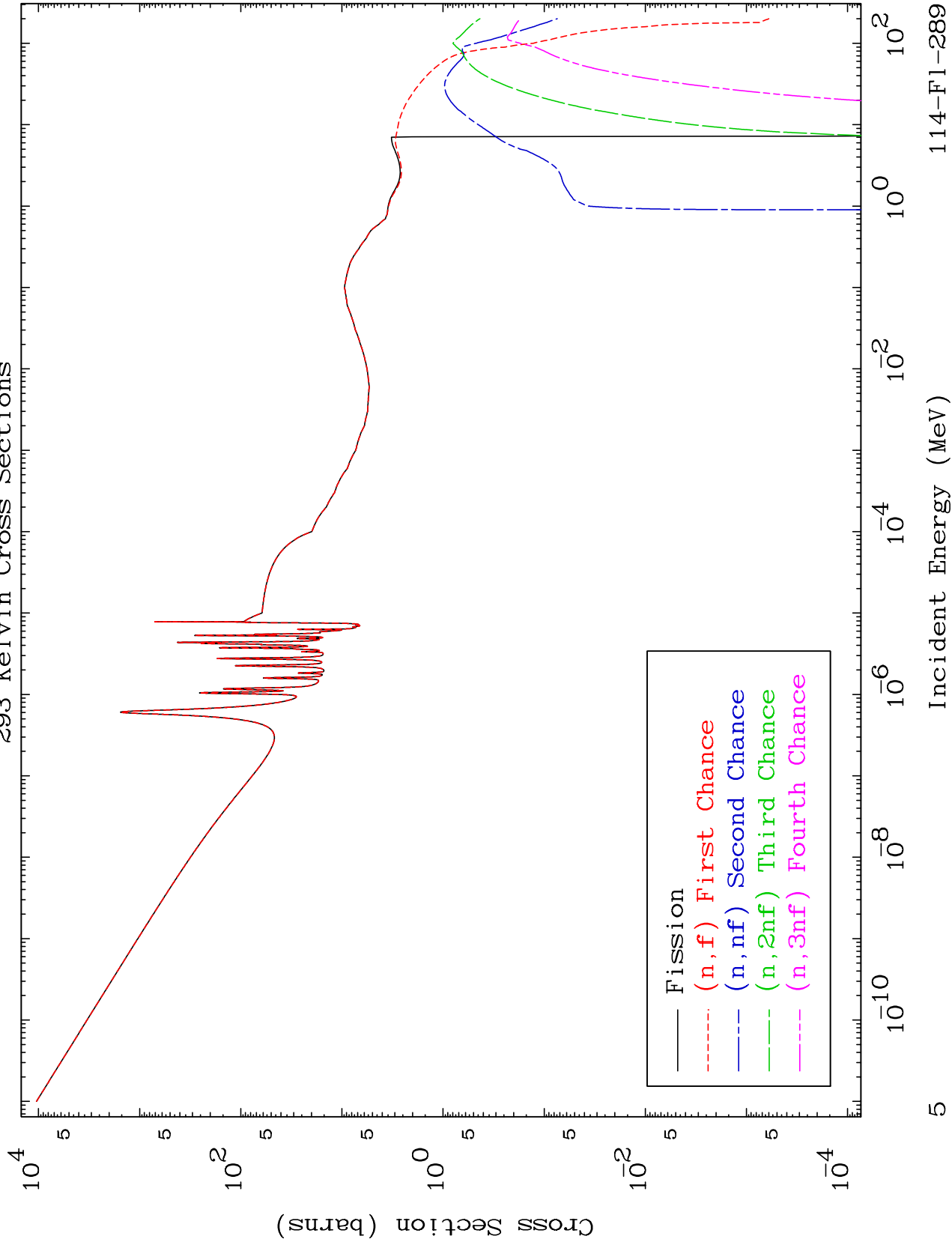




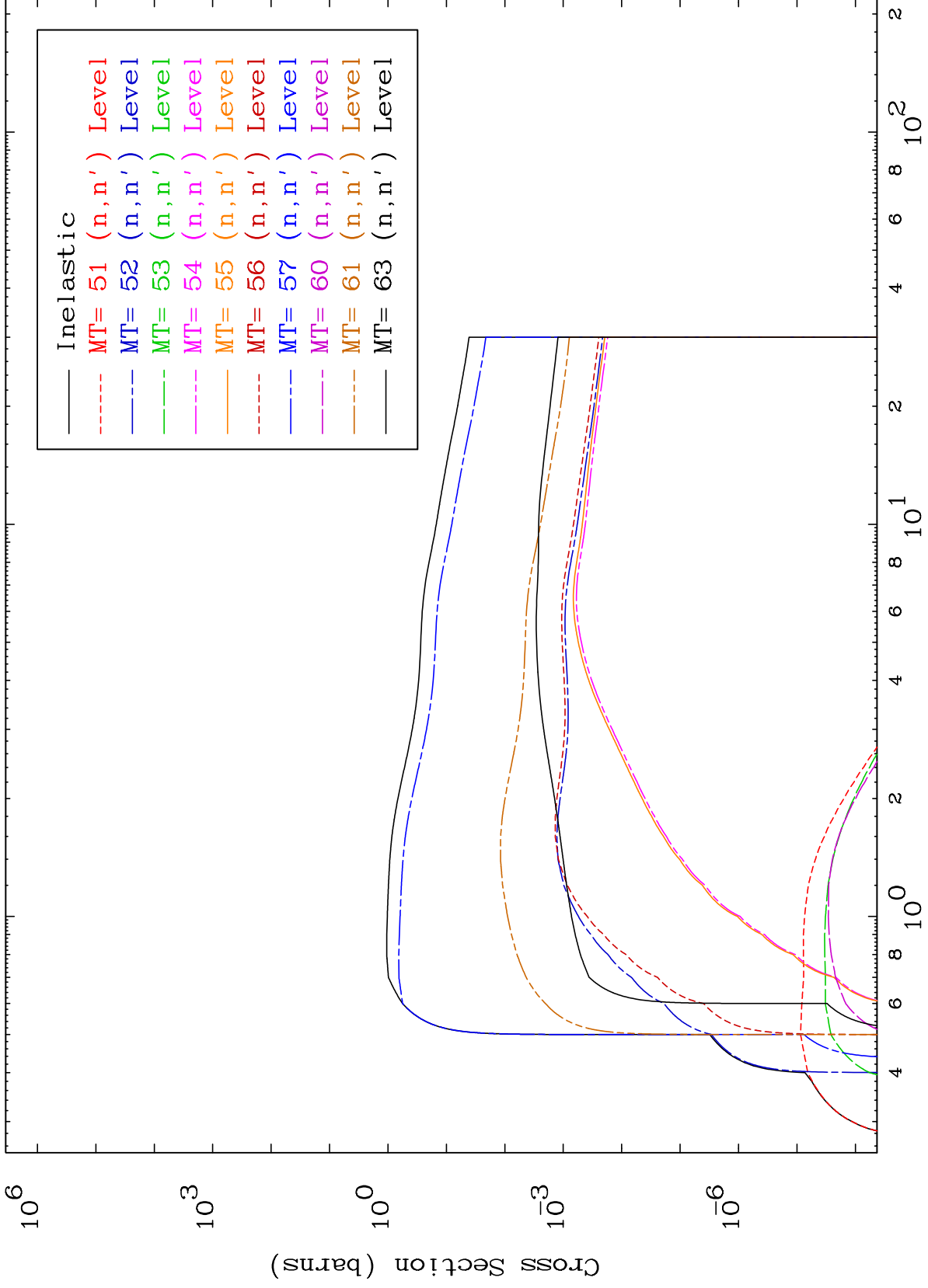
MAT 1489

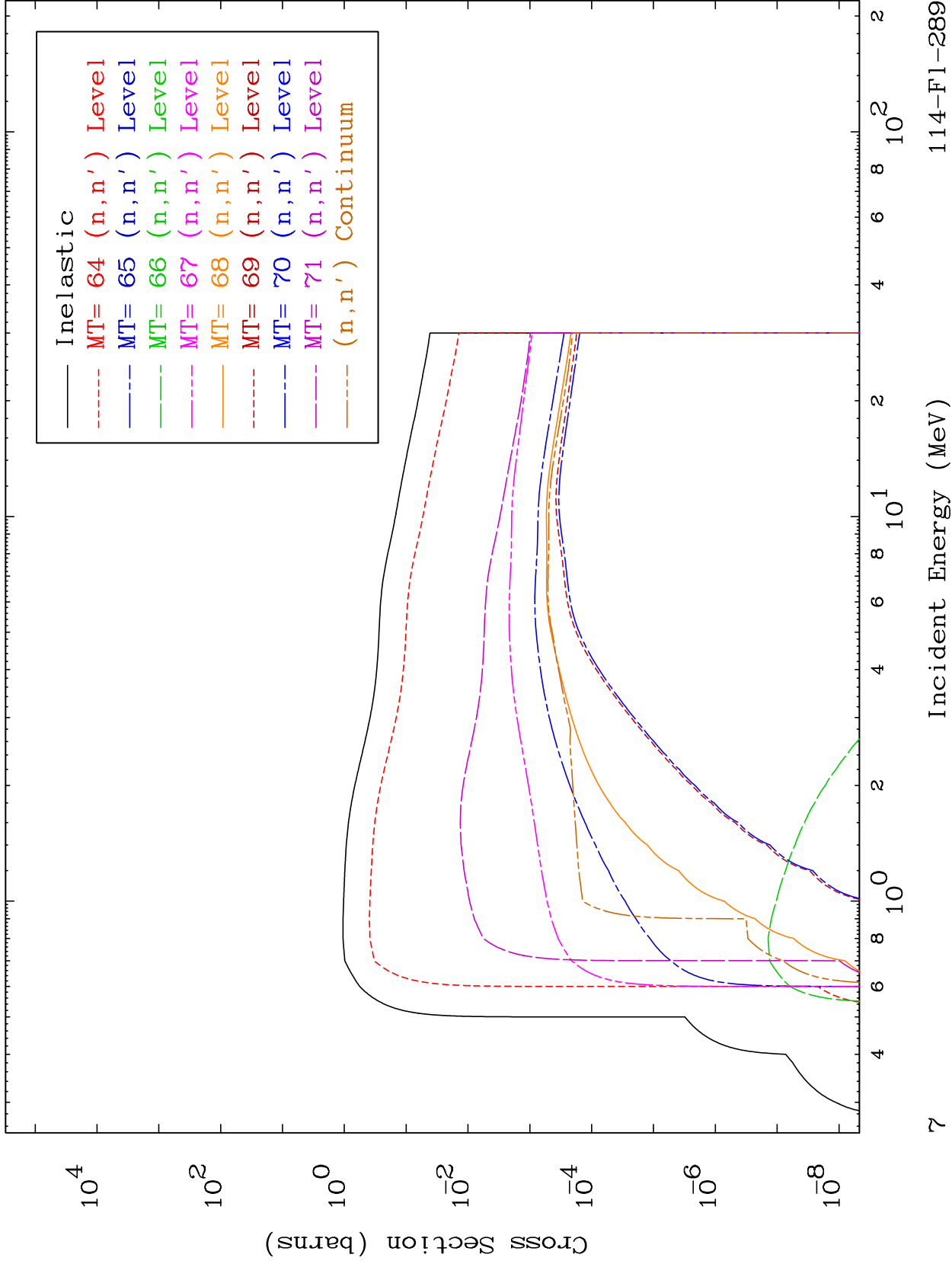
Fission  
293 Kelvin Cross Sections

114-FI-289



293 Kelvin Cross Sections



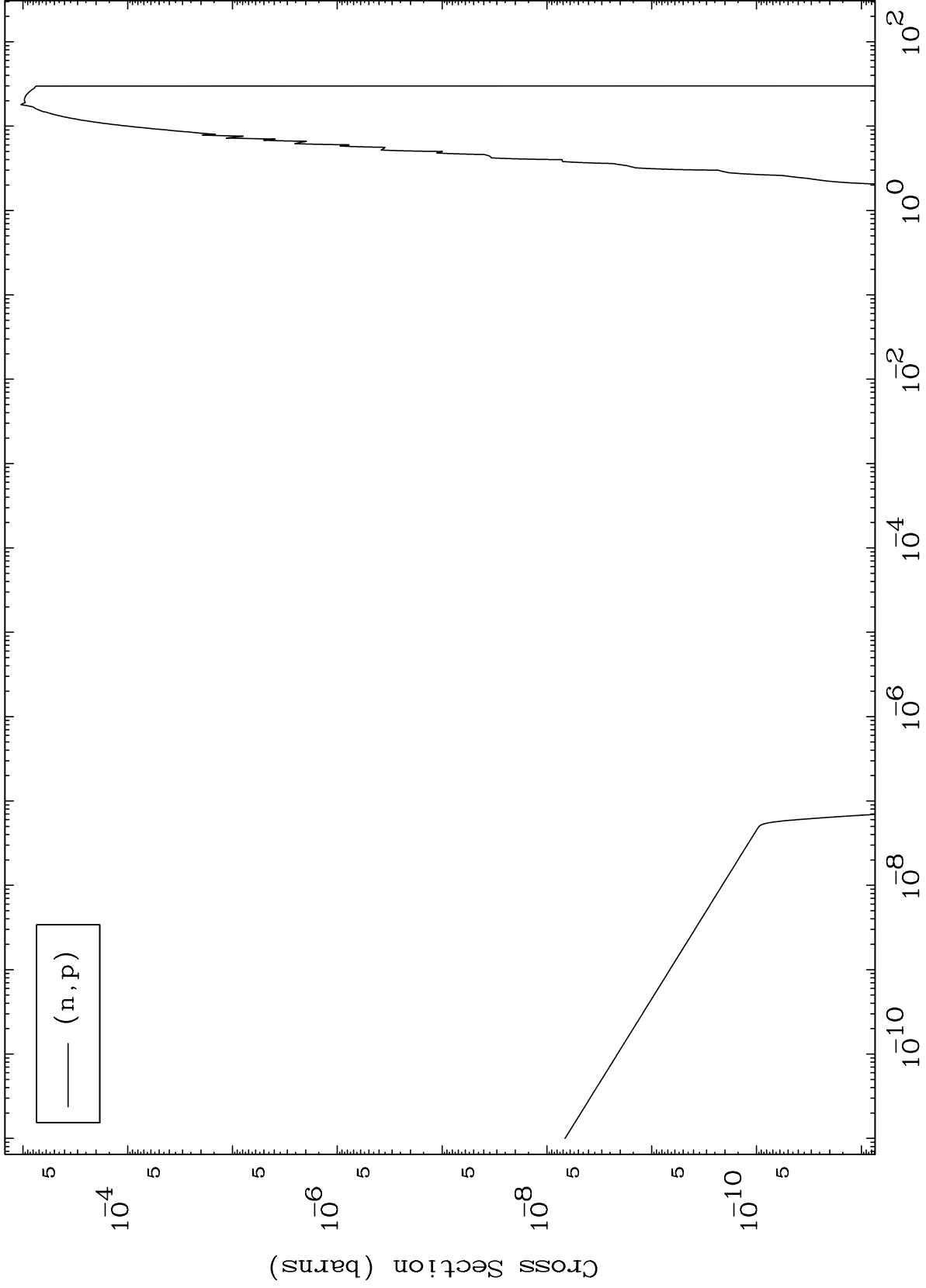




MAT 1489

(n,p) Levels  
293 Kelvin Cross Sections

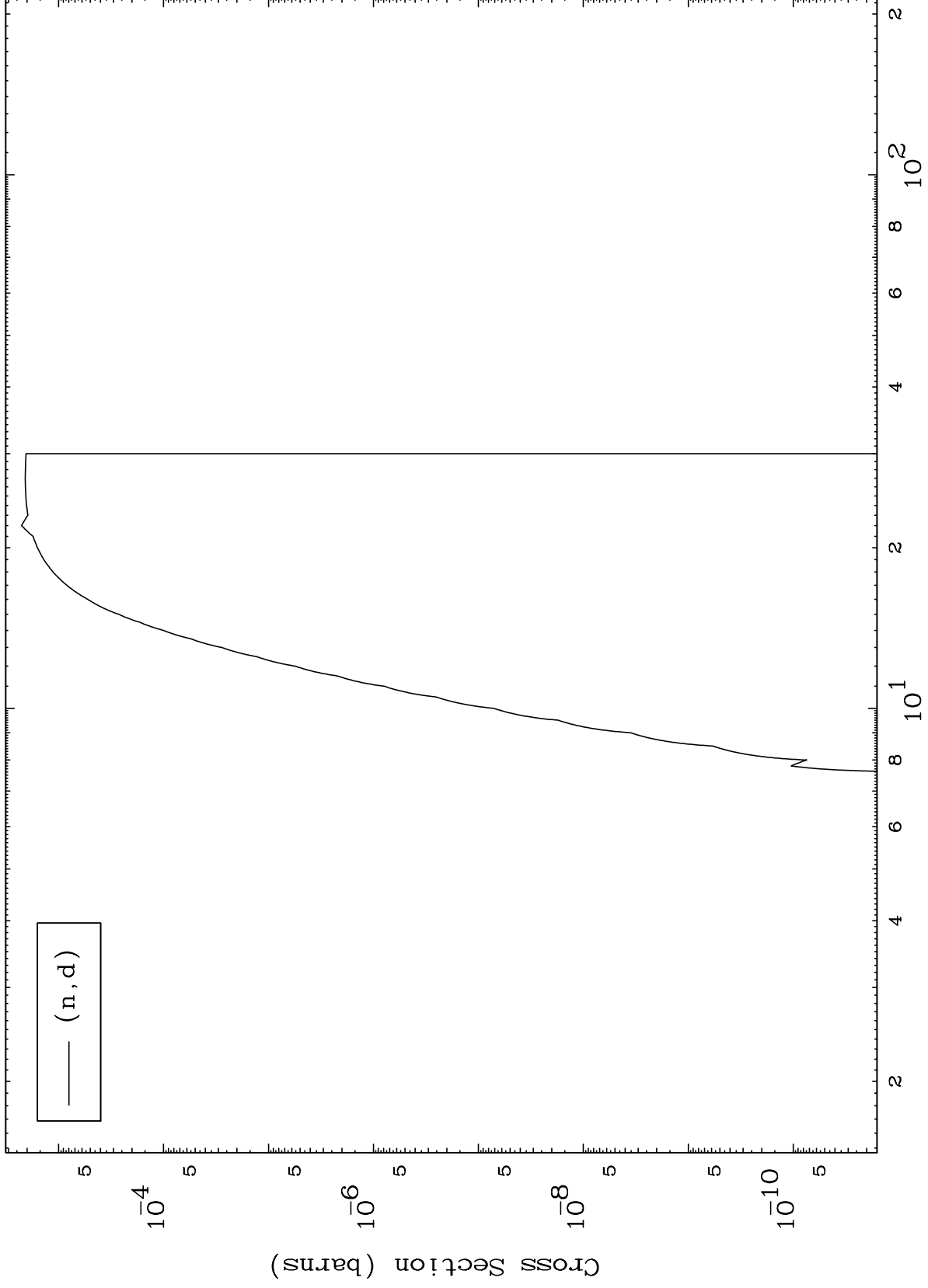
114-F1-289



MAT 1489

(n,d) Levels  
293 Kelvin Cross Sections

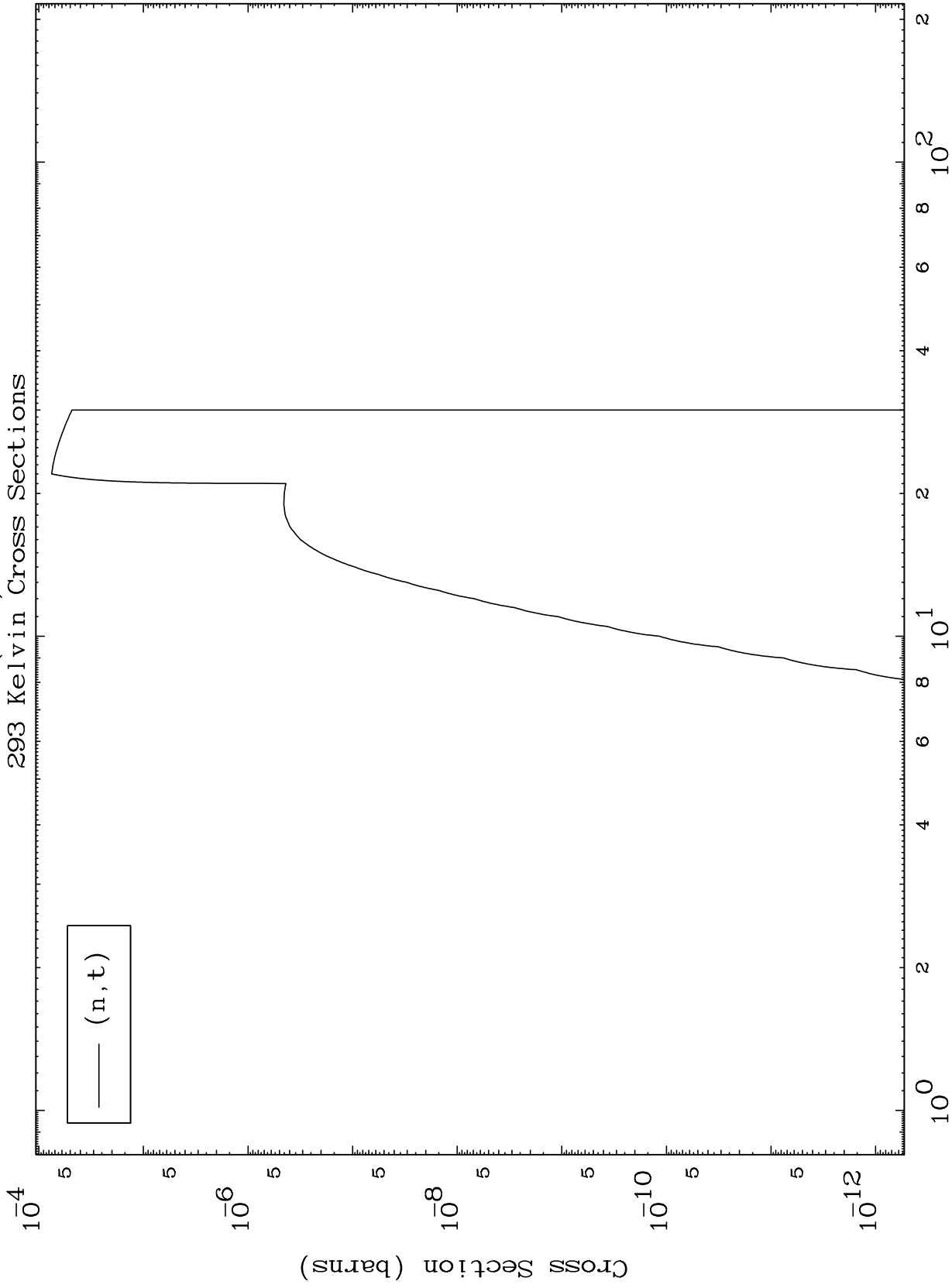
114-F1-289



MAT 1489

(n,t) Levels  
293 Kelvin Cross Sections

114-F1-289



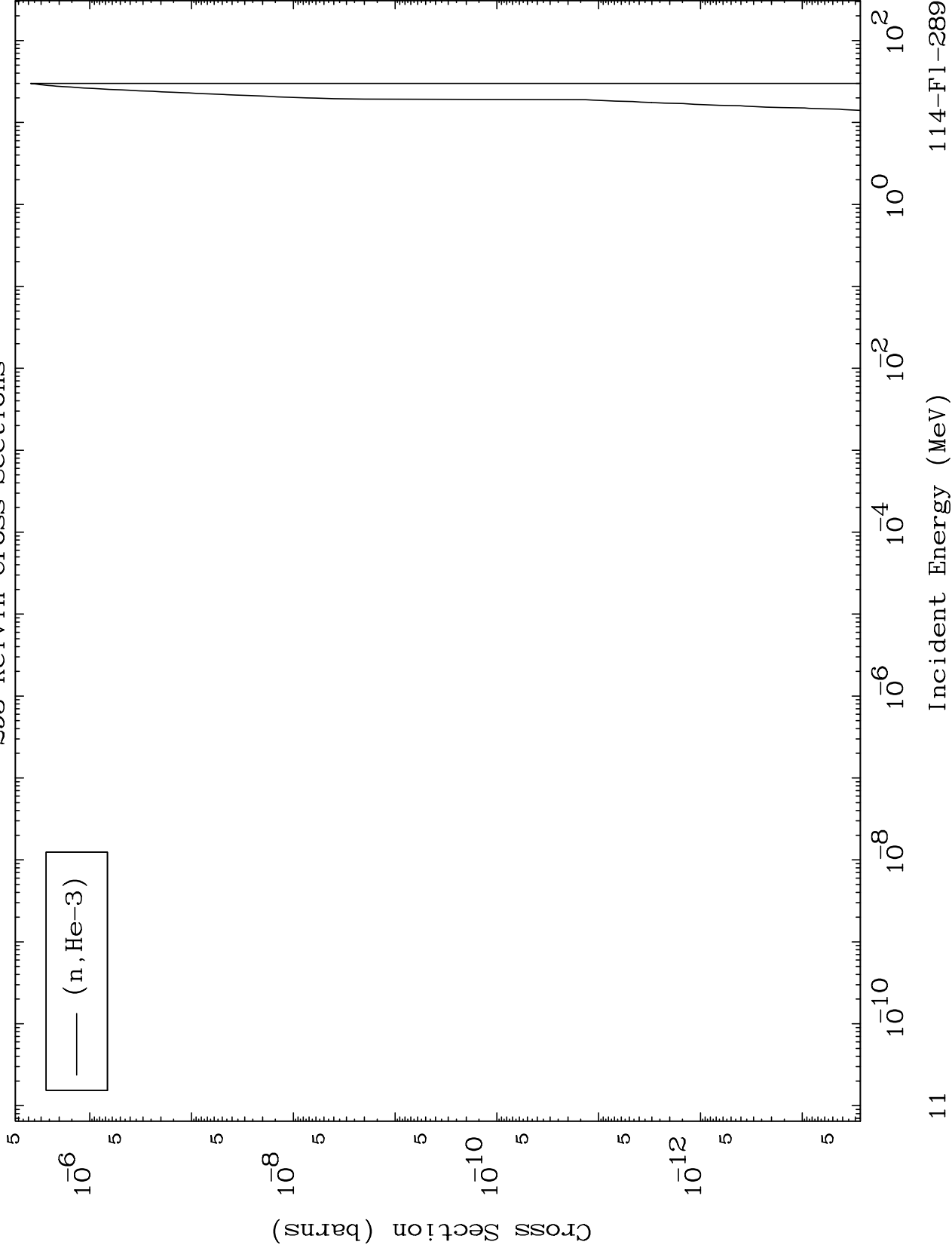
Incident Energy (MeV)

114-F1-289

MAT 1489

(n,He3) Levels  
293 Kelvin Cross Sections

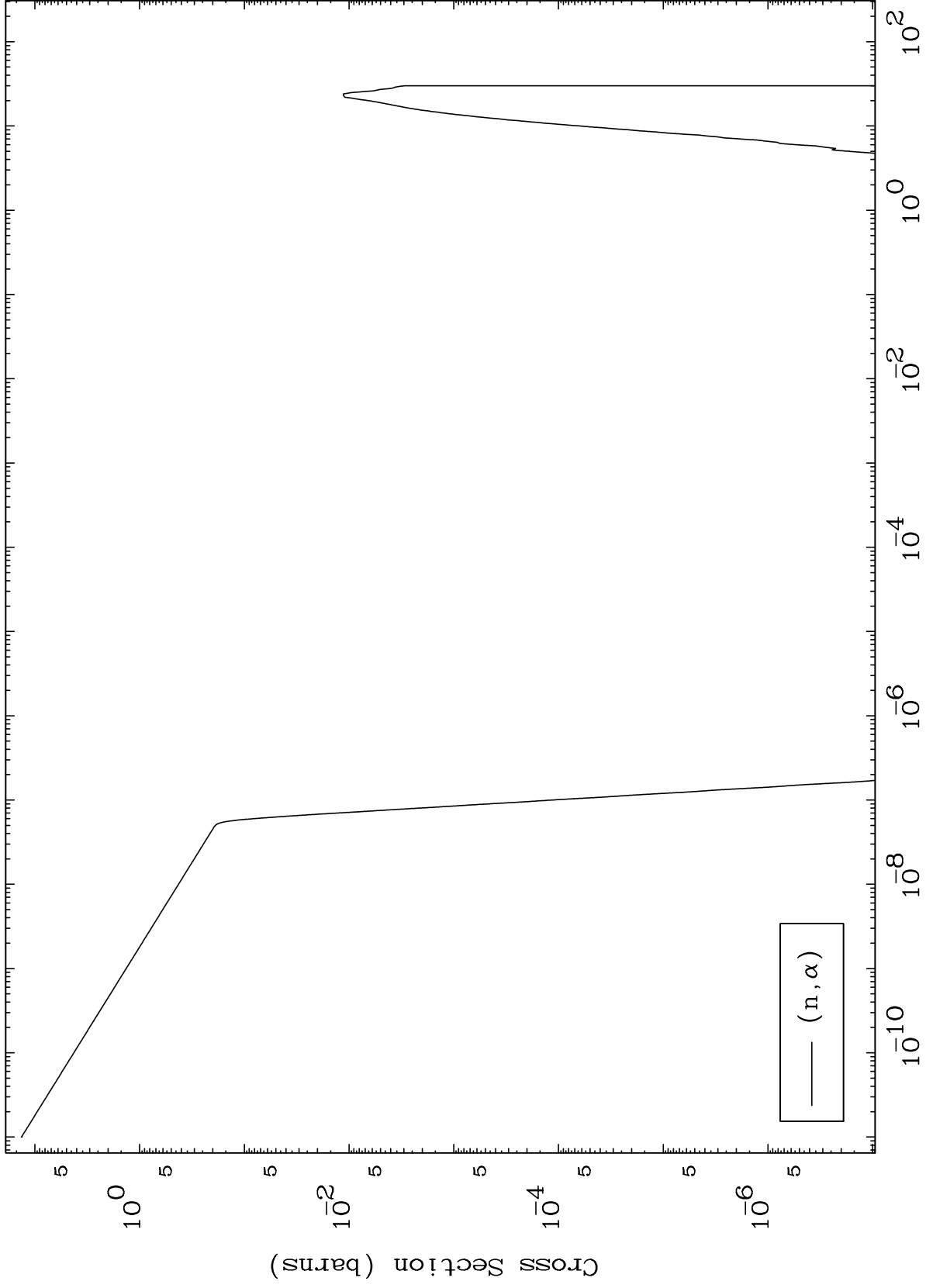
114-F1-289



MAT 1489

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

114-F1-289



12

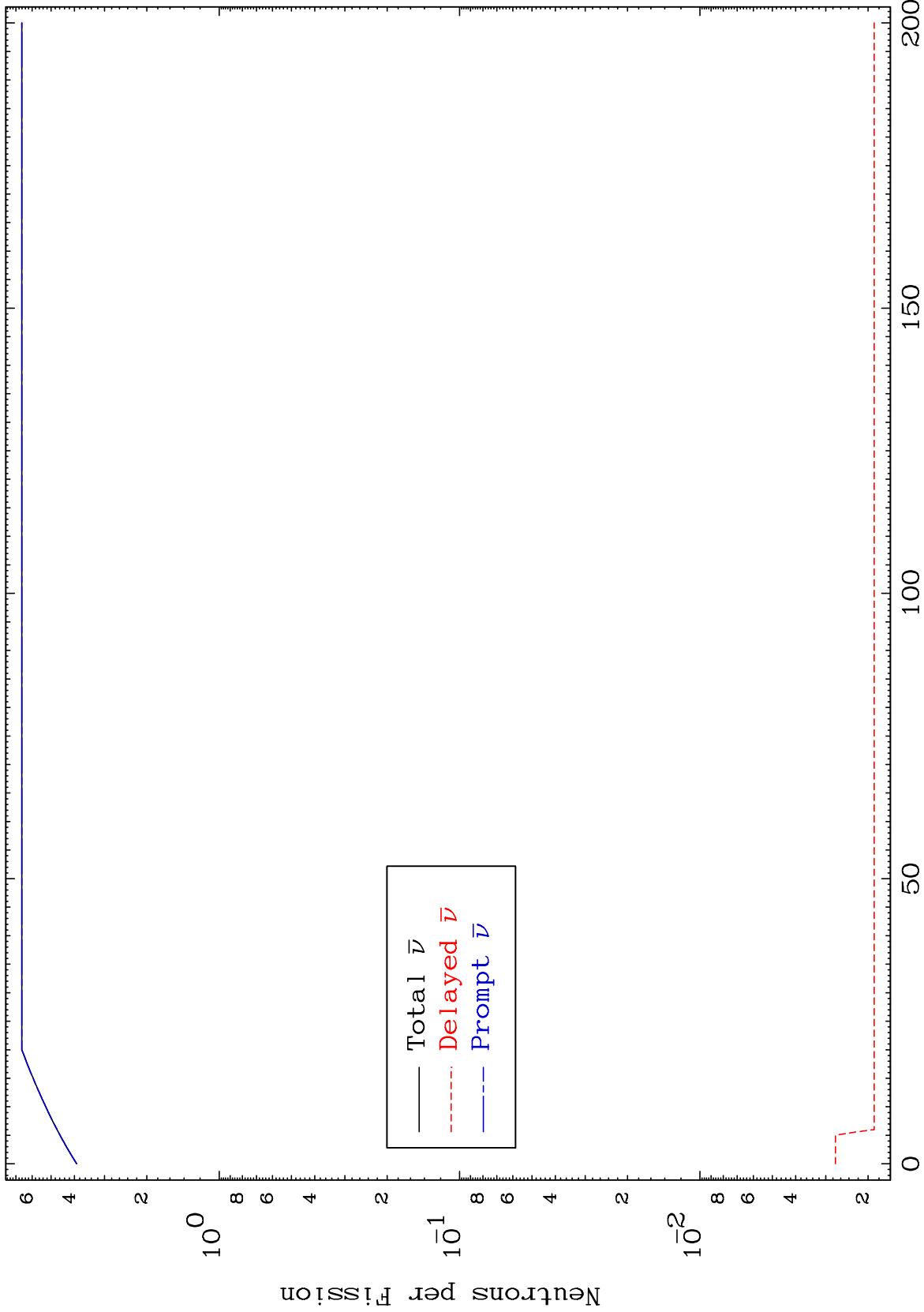
Incident Energy (MeV)

114-F1-289

MAT 1489

114-F1-289

Neutrons  
per Fission



13

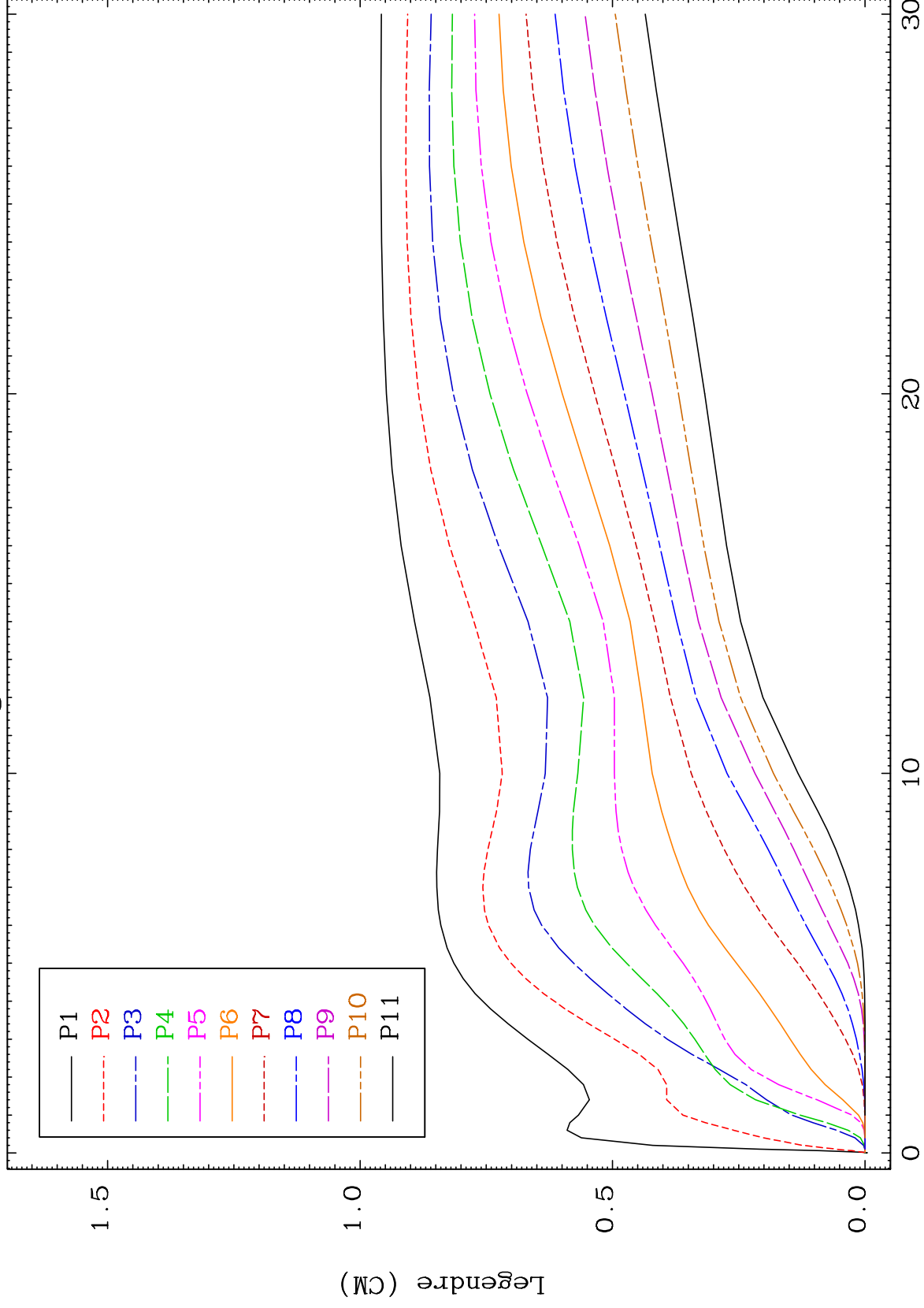
Incident Energy (MeV)

114-F1-289

MAT 1489

Elastic Legendre Coefficients

114-F1-289



14

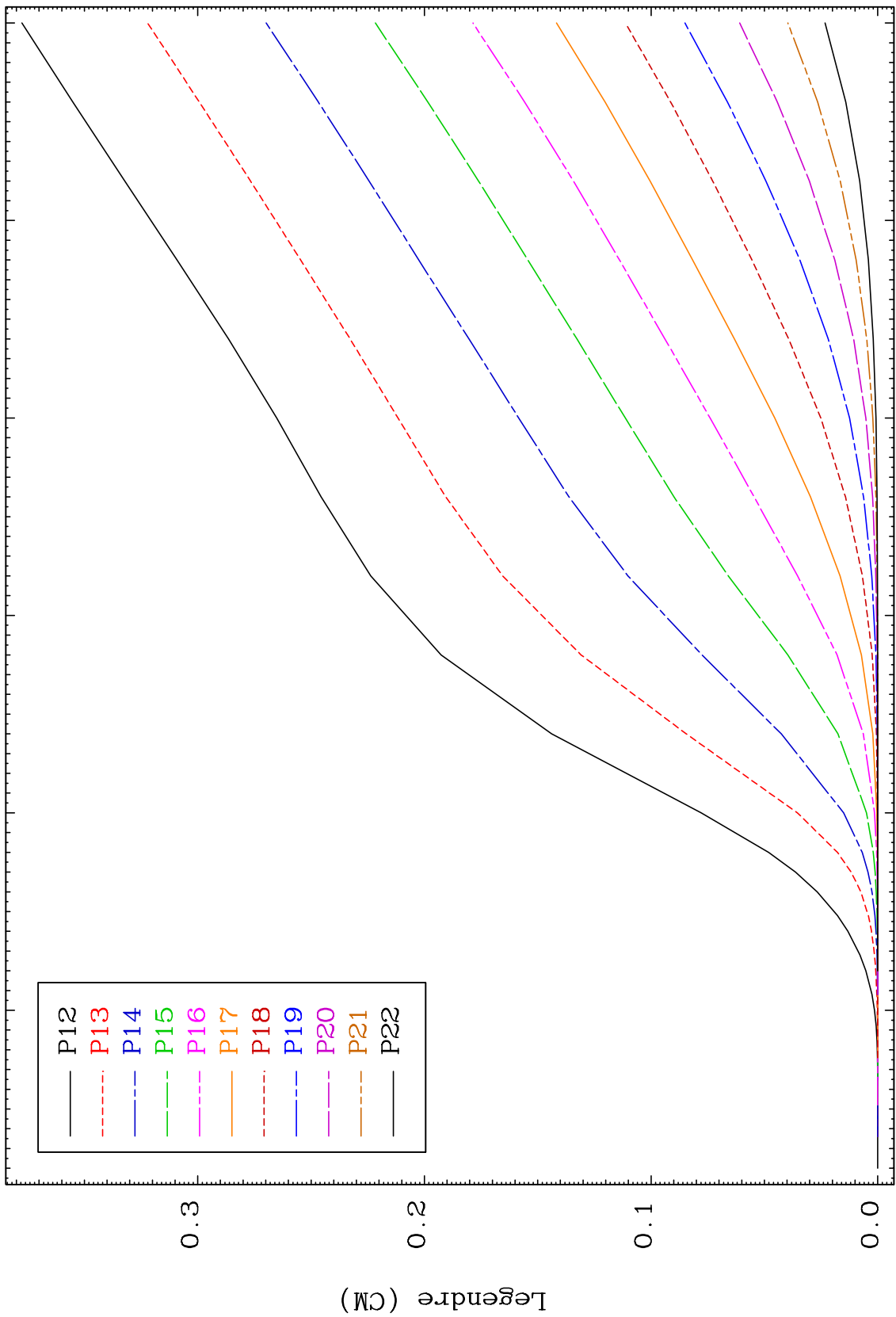
Incident Energy (MeV)

114-F1-289

MAT 1489

Elastic Legendre Coefficients

114-F1-289



15

Incident Energy (MeV)

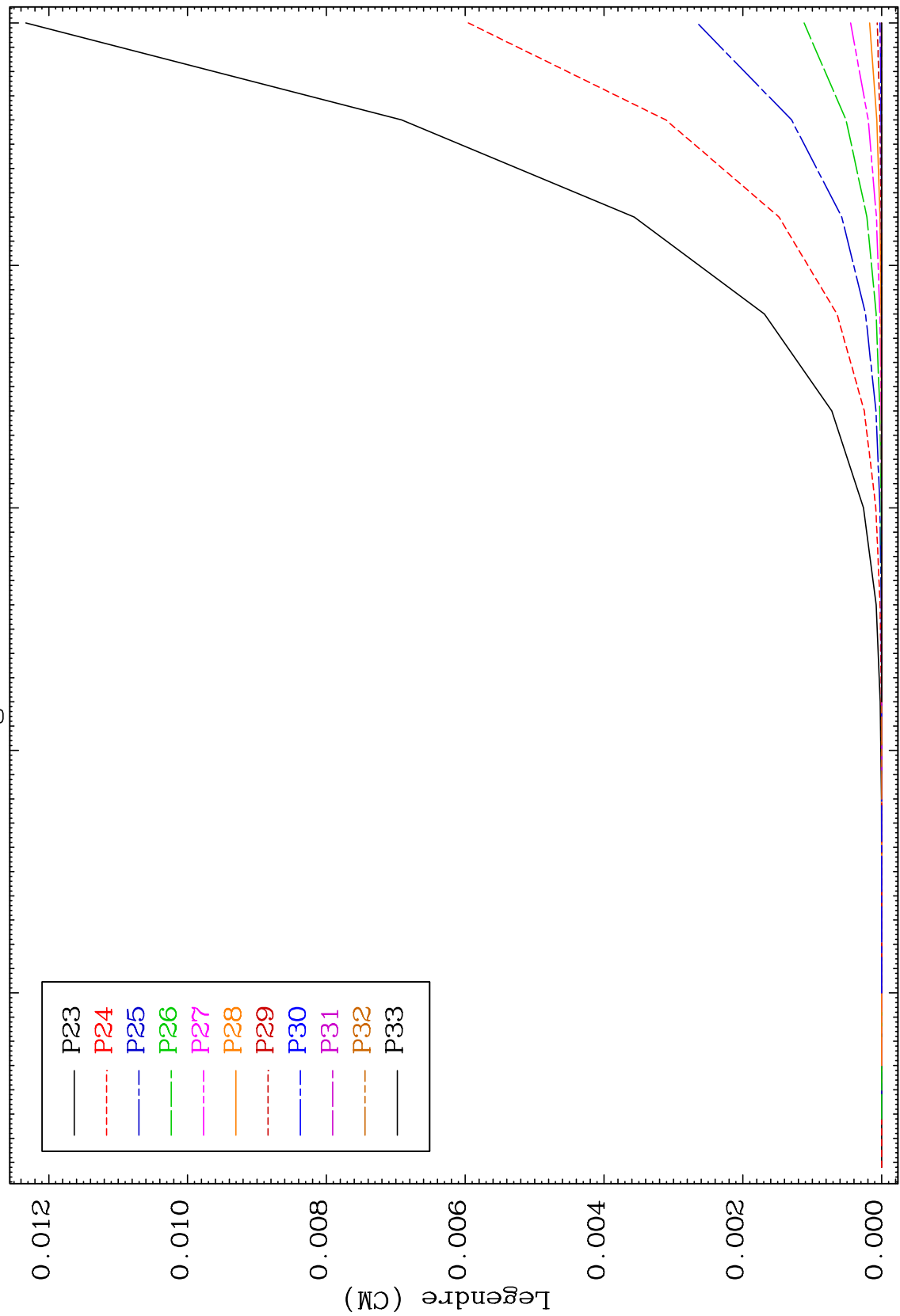
114-F1-289



MAT 1489

Elastic Legendre Coefficients

114-F1-289



16

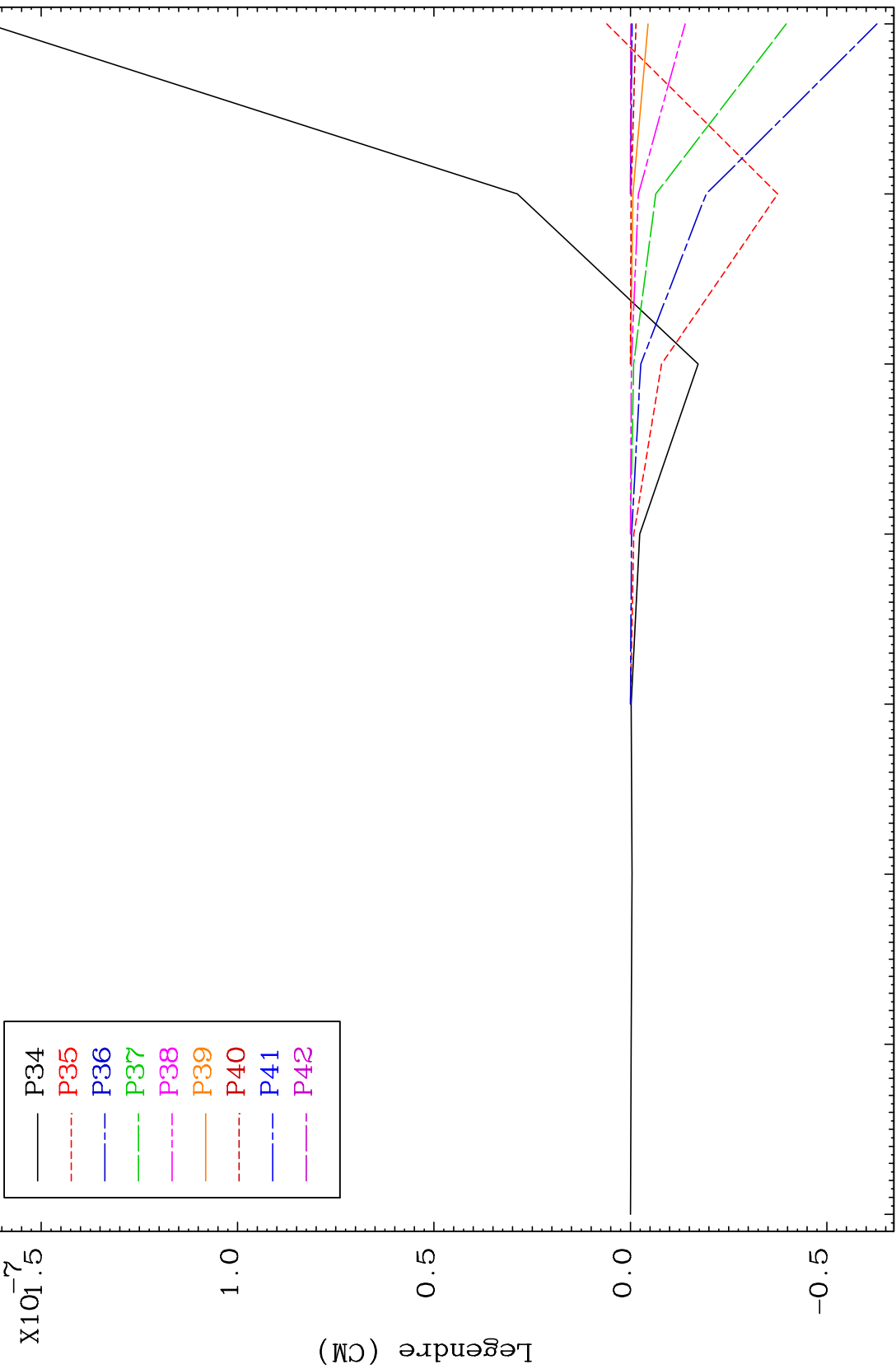
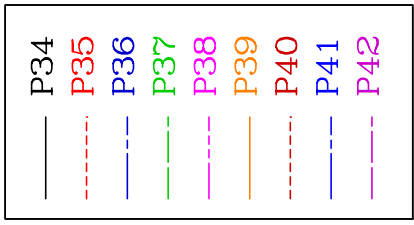
Incident Energy (MeV)

114-F1-289

MAT 1489

Elastic Legendre Coefficients

114-F1-289

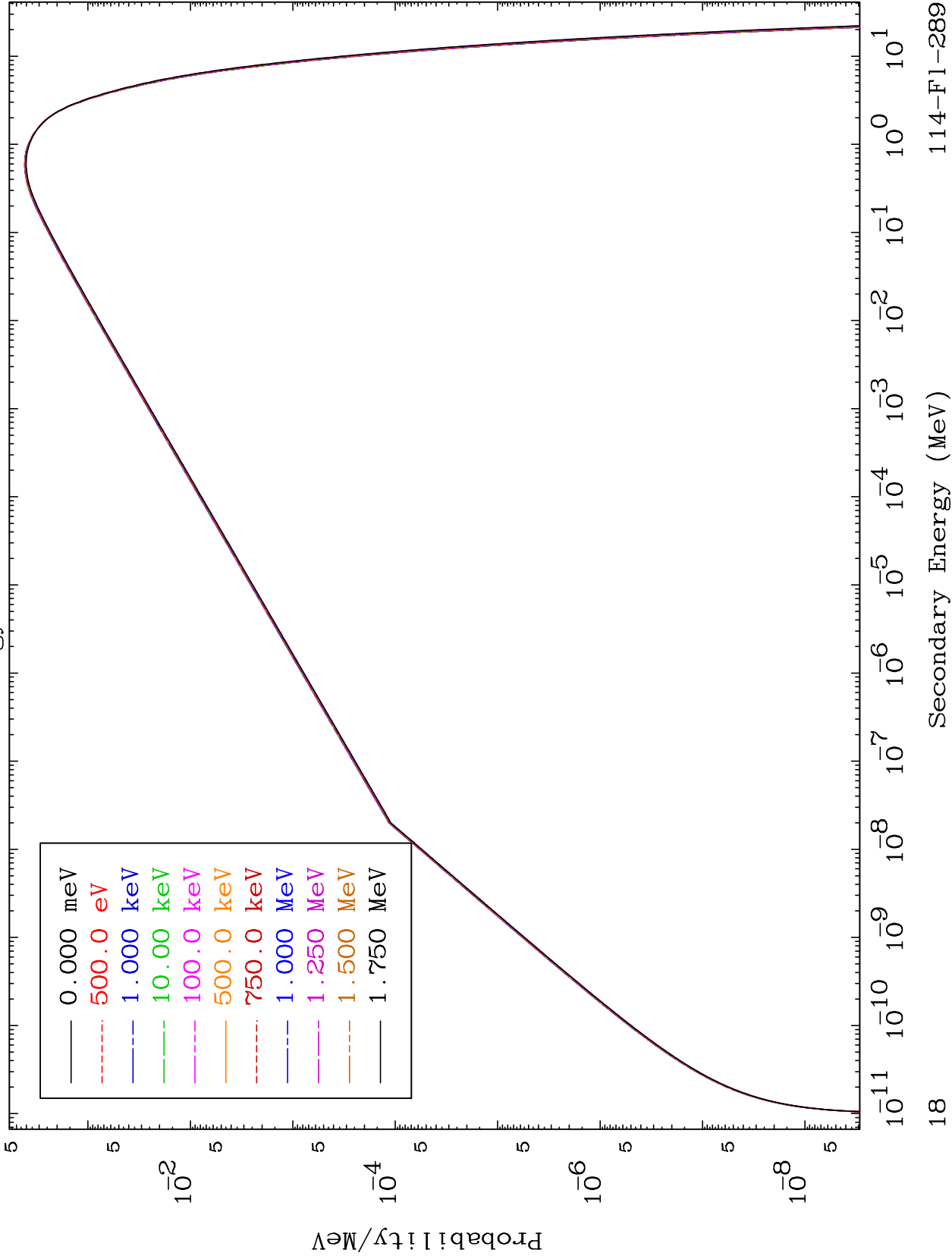


17

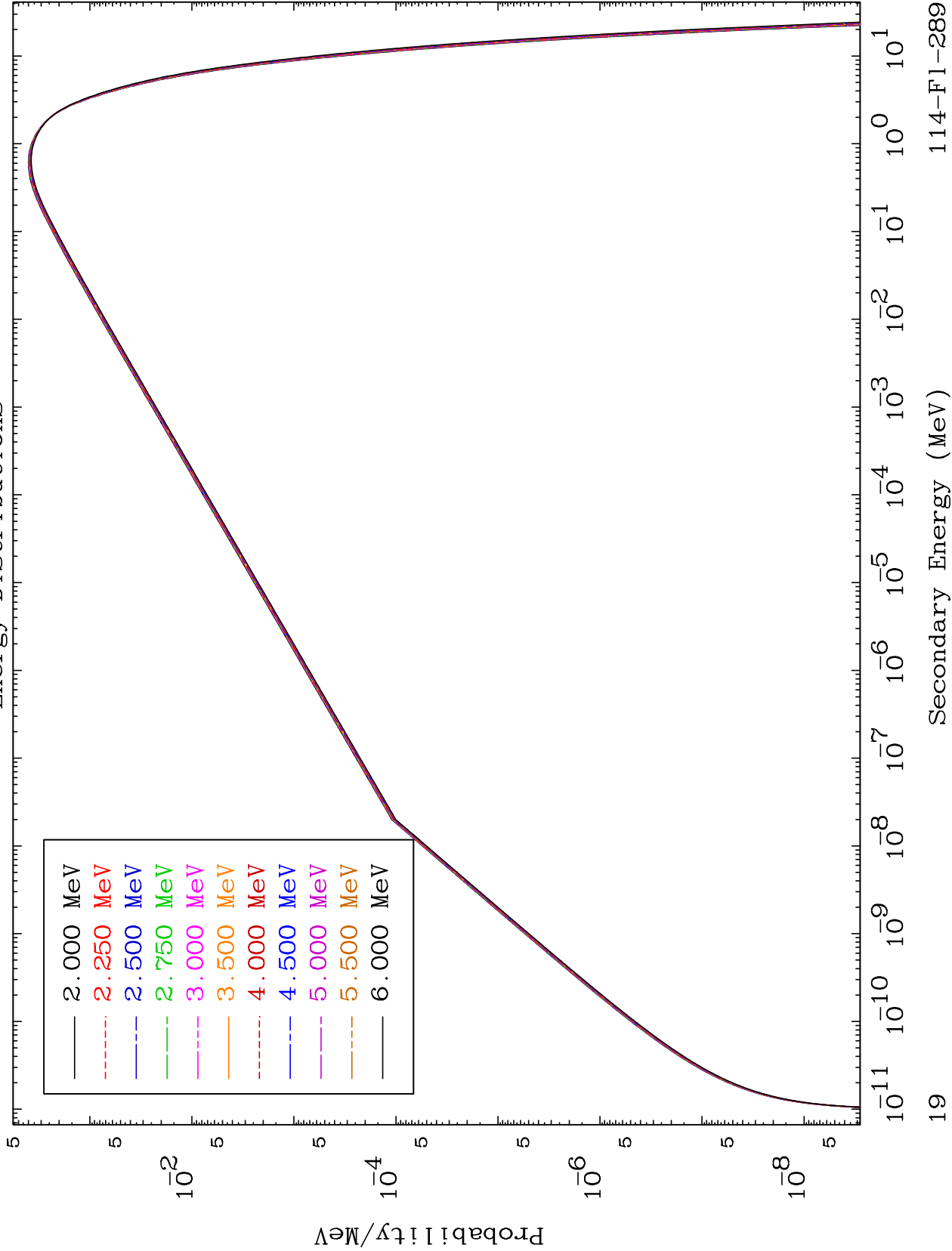
Incident Energy (MeV)

114-F1-289

Fission Energy Distributions



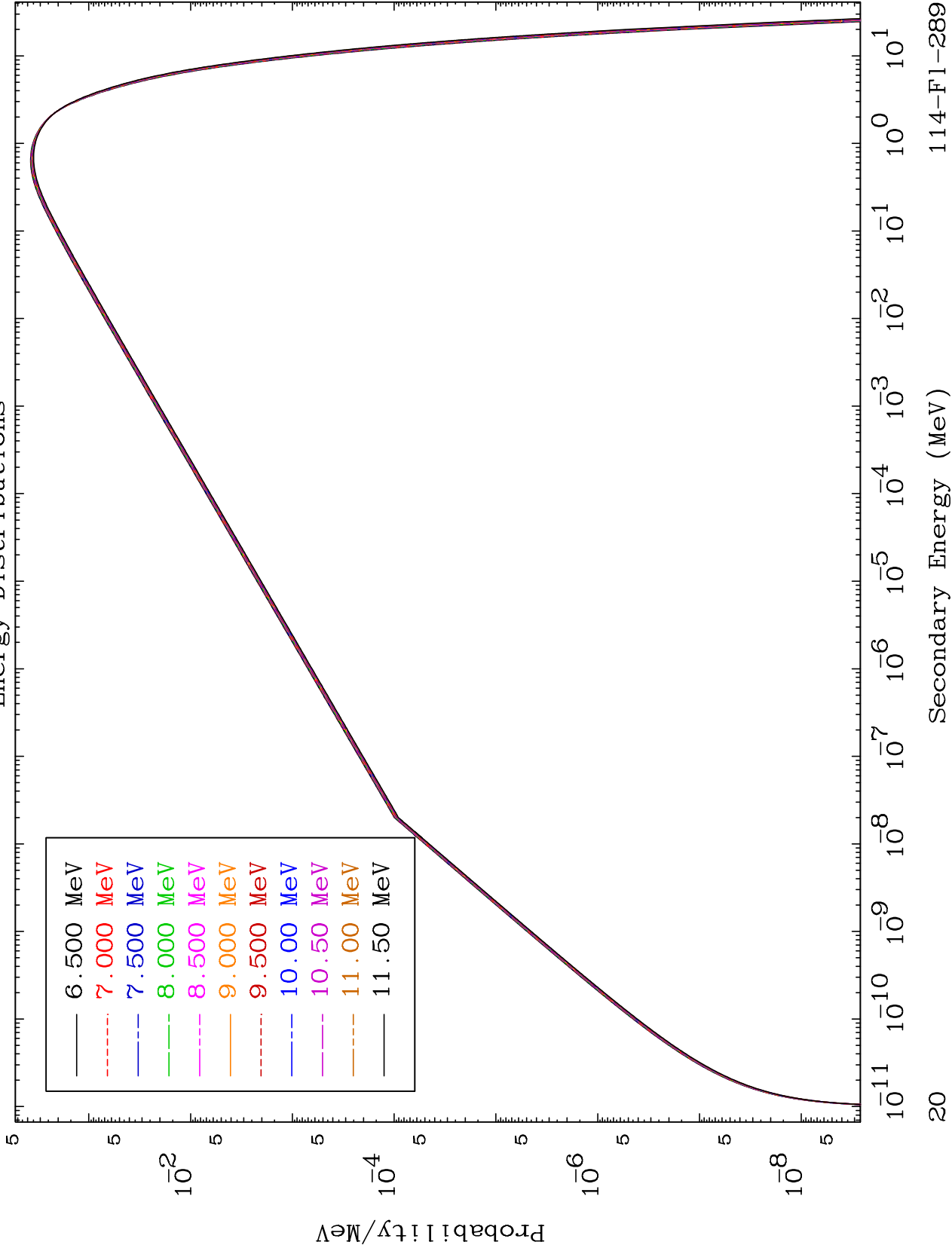
Fission  
Energy Distributions



MAT 1489

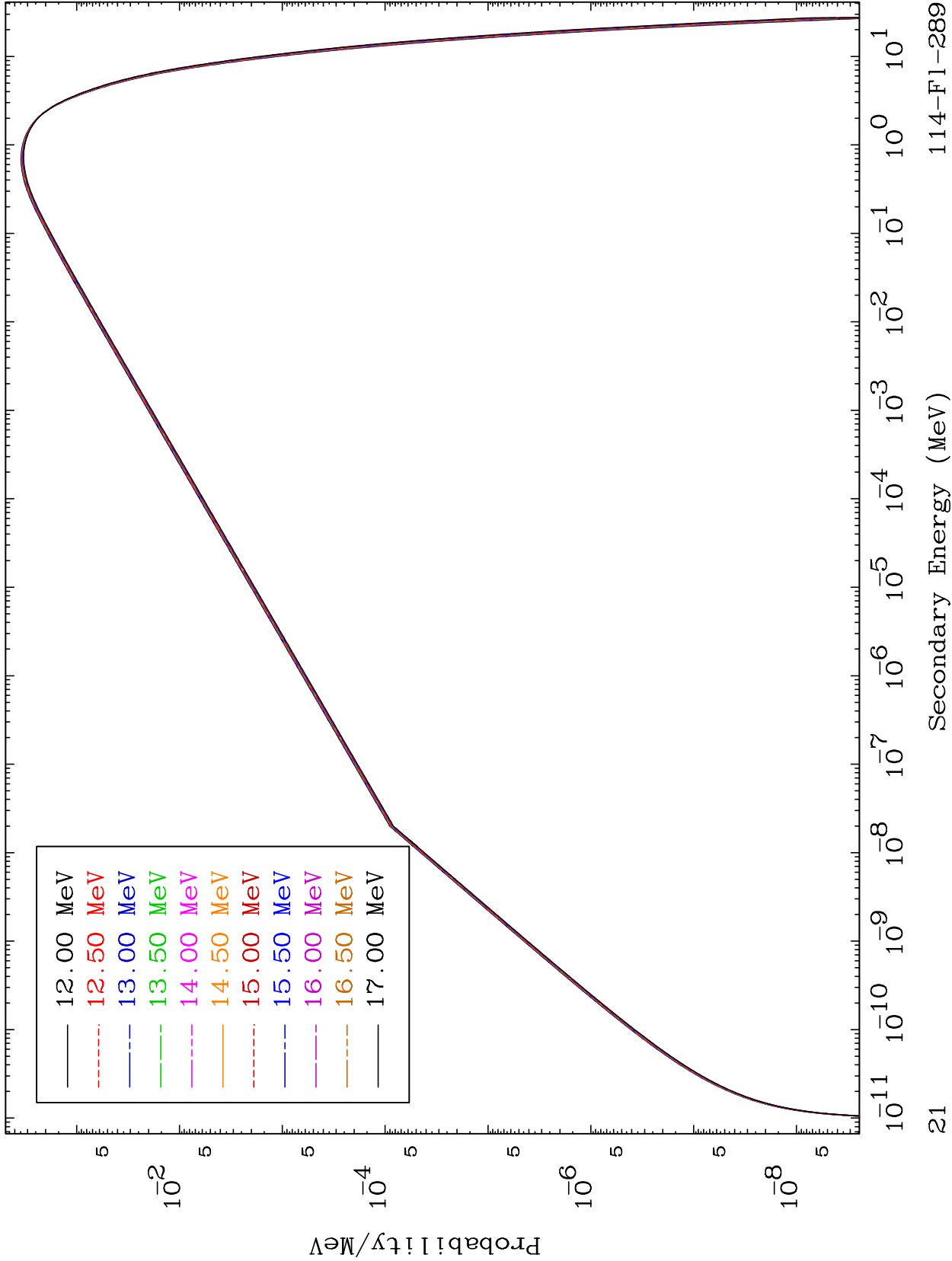
### Fission Energy Distributions

114-F1-289



114-F1-289

Secondary Energy (MeV)



Fission Energy Distributions

