

Program EVALPLOT
(Version 2021-1)

by

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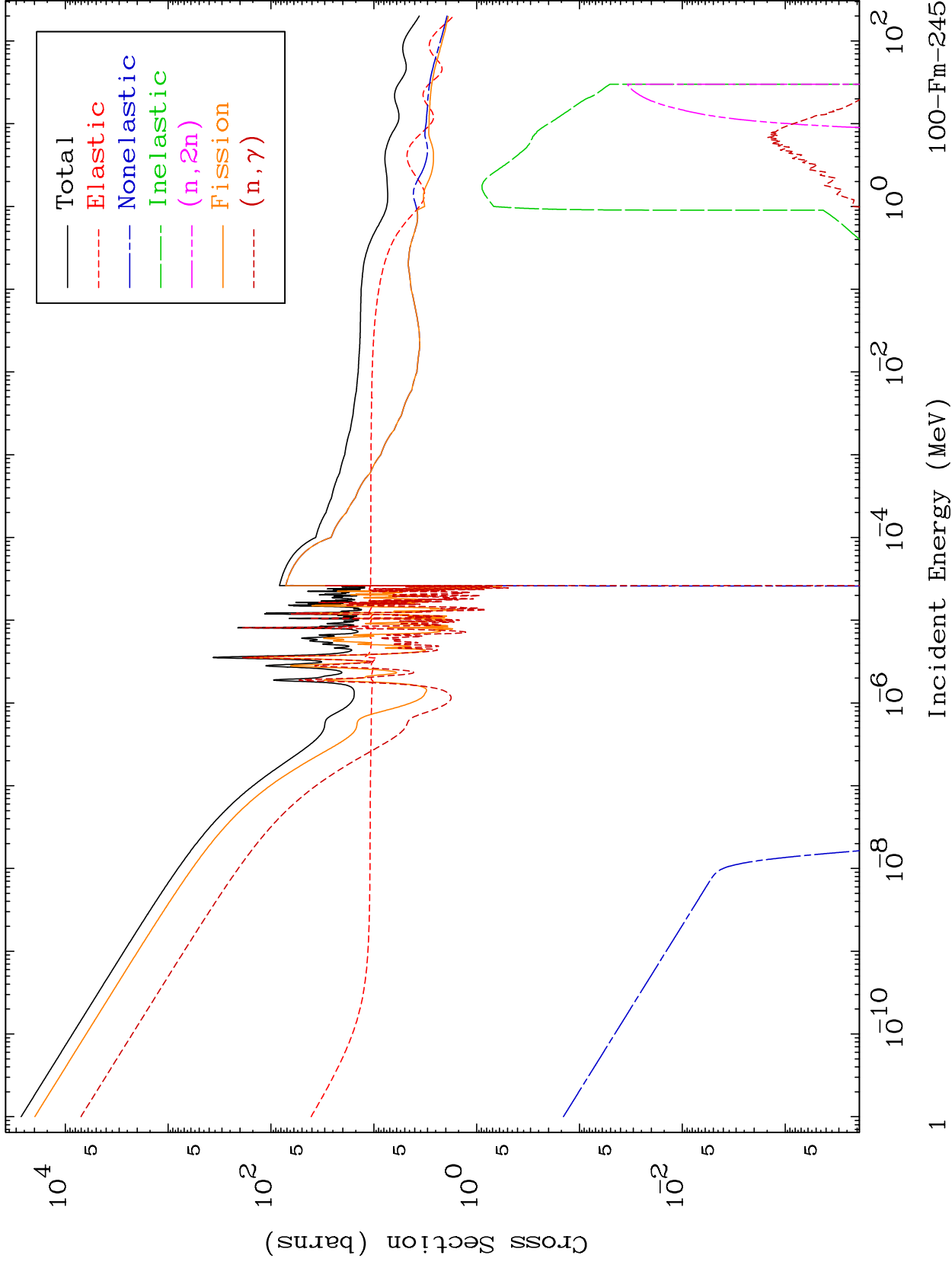
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9926

Neutron Major
293 Kelvin Cross Sections

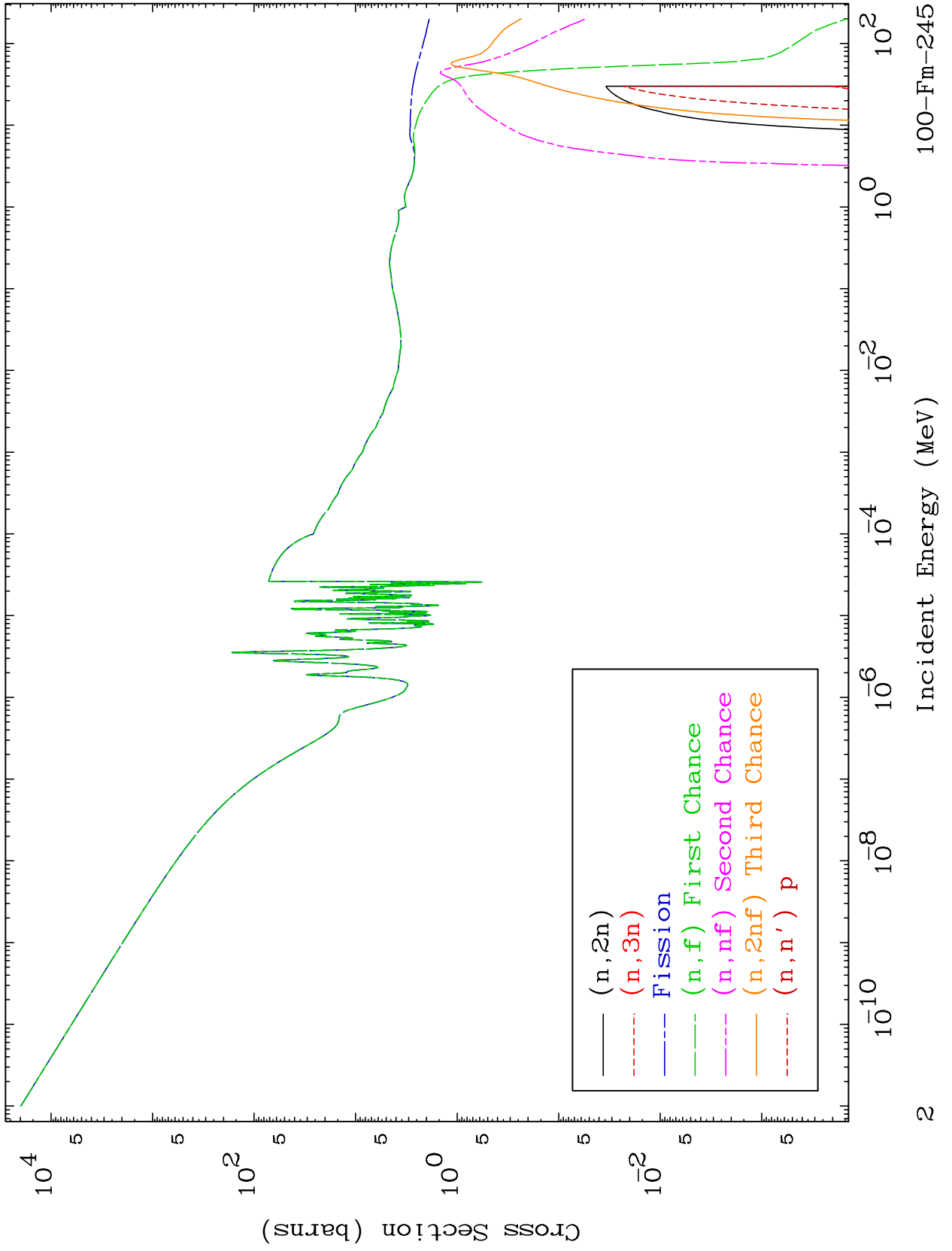
100-Fm-245



MAT 9926

Neutron Absorption
293 Kelvin Cross Sections

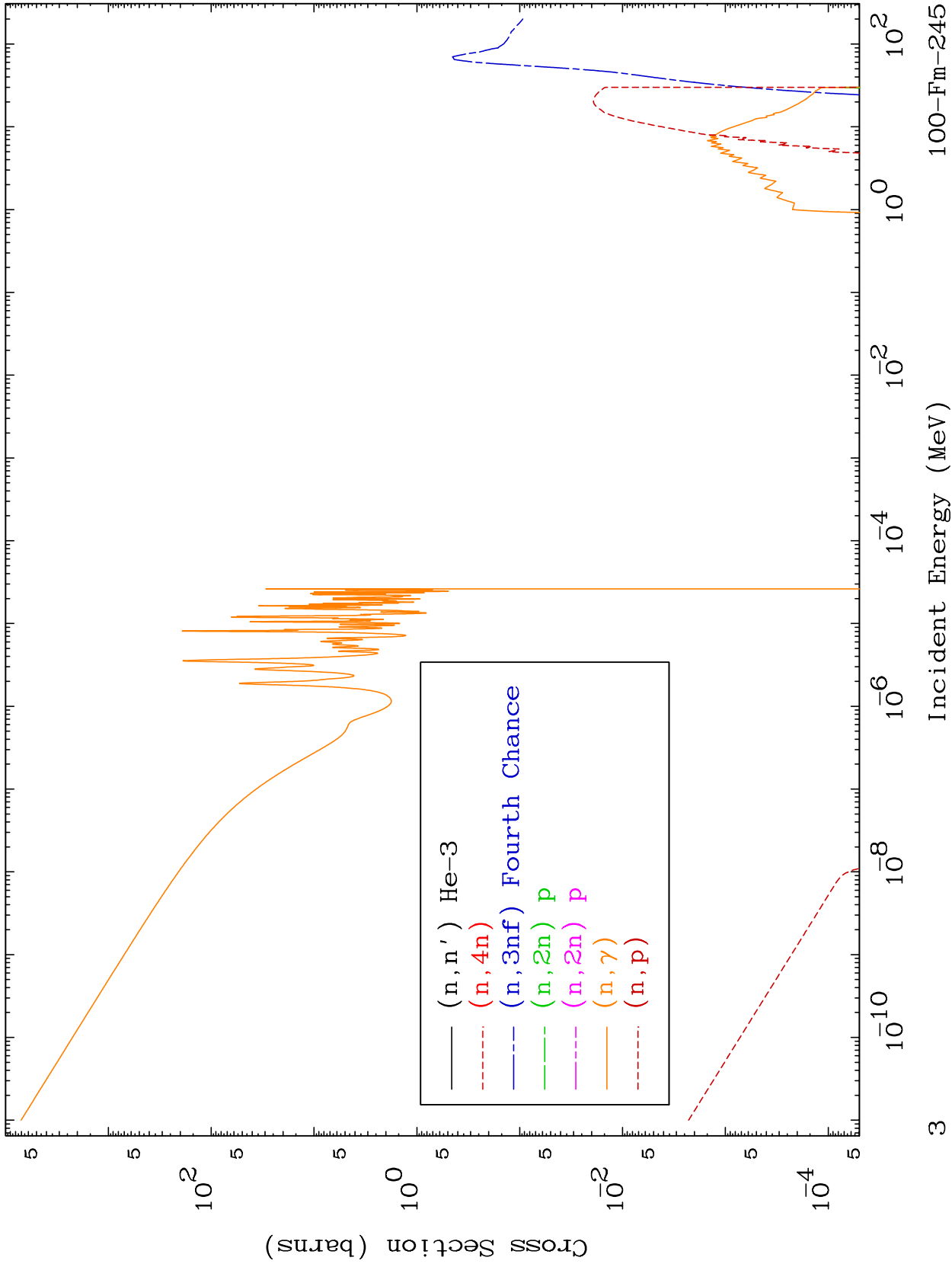
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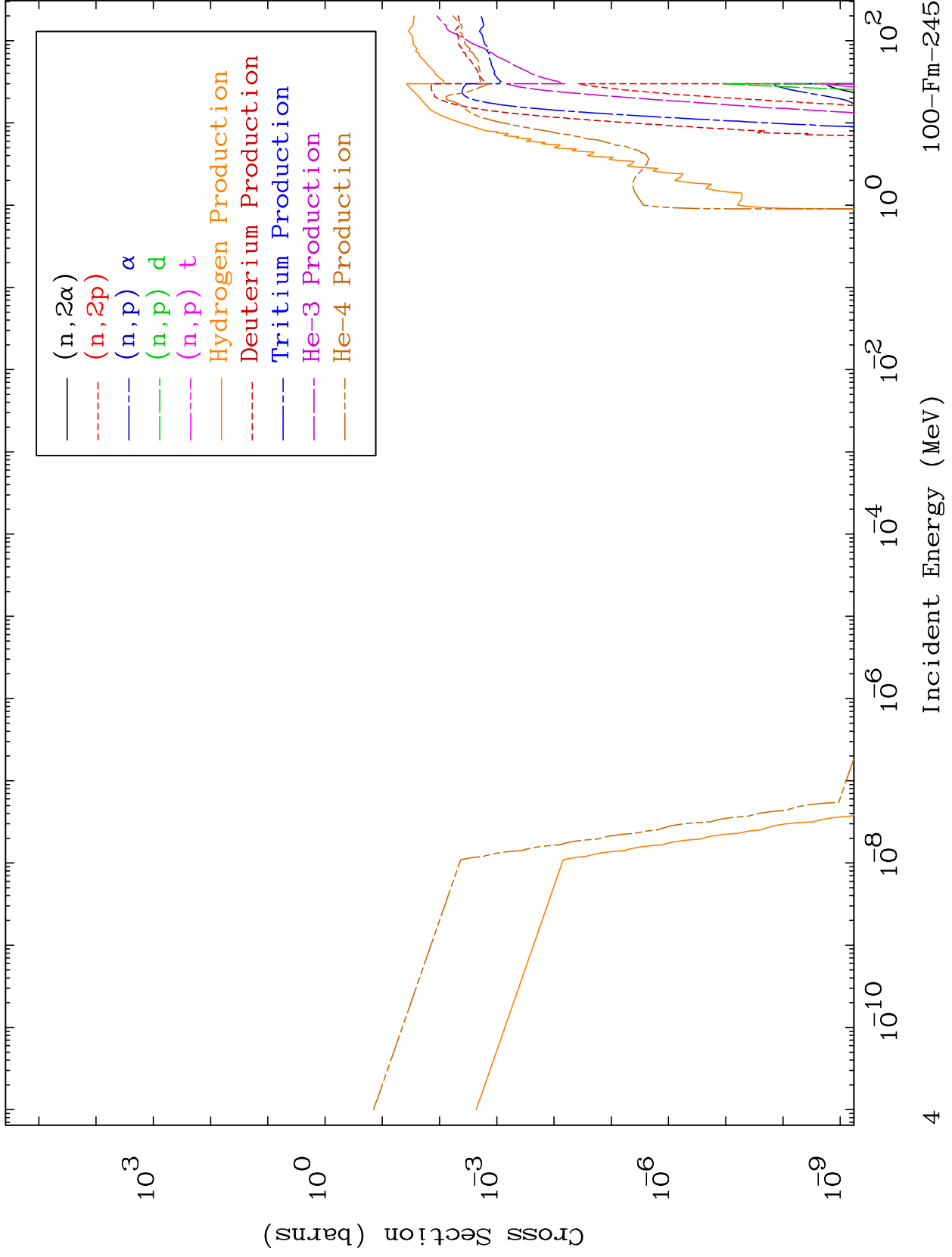


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Neutron Absorption
293 Kelvin Cross Sections

100-Fm-245

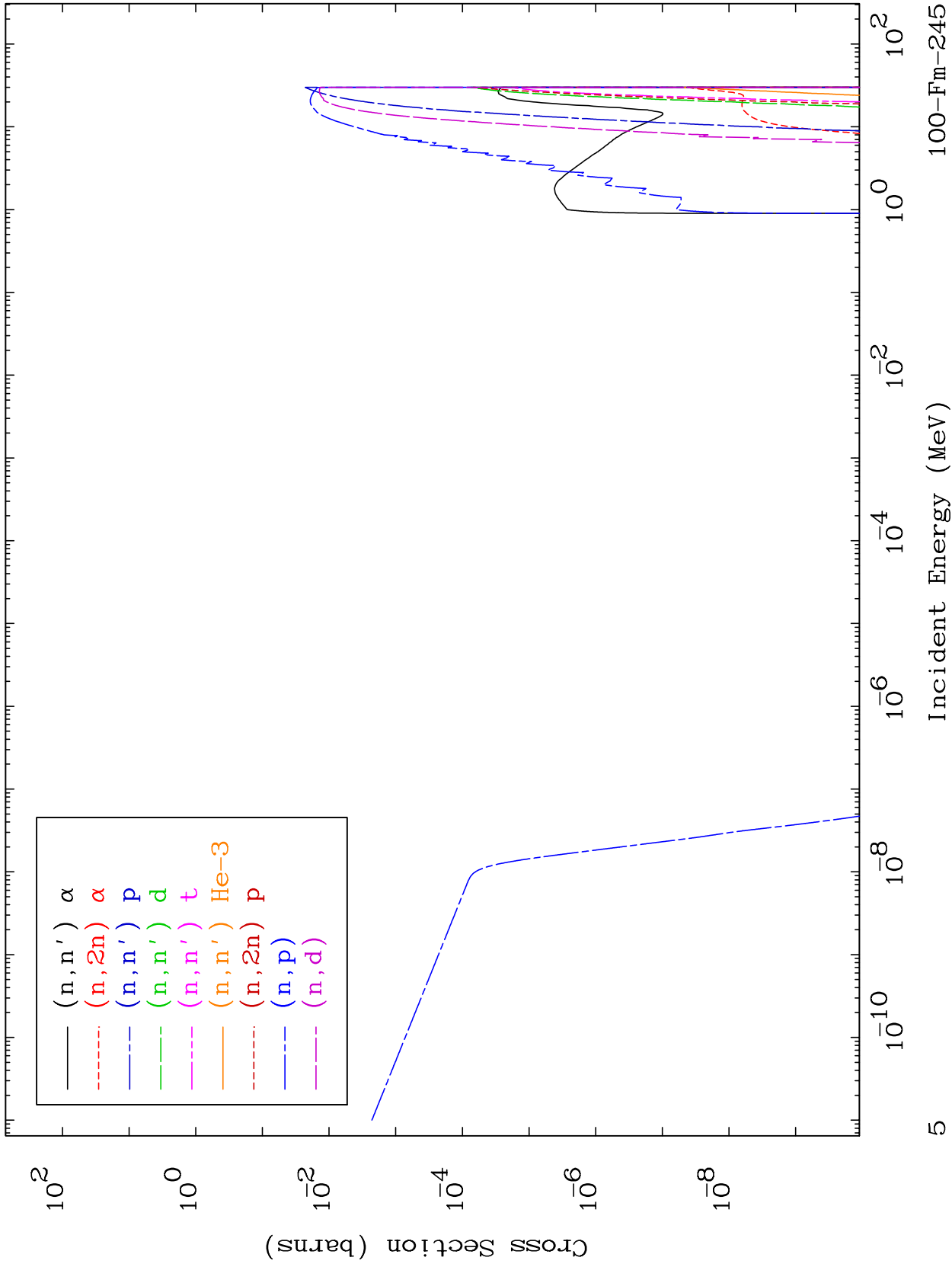




MAT 9926

Charged Particle
293 Kelvin Cross Sections

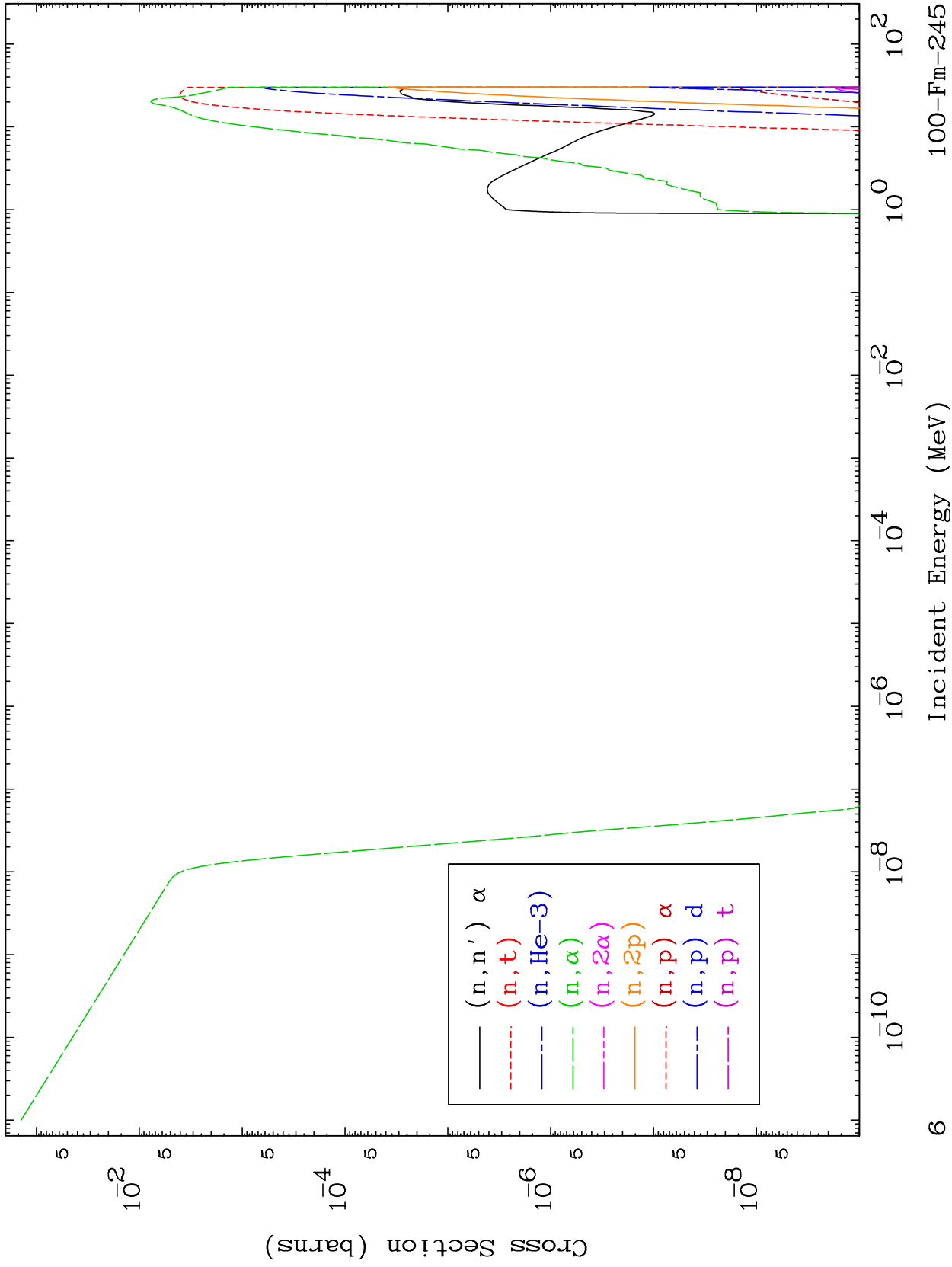
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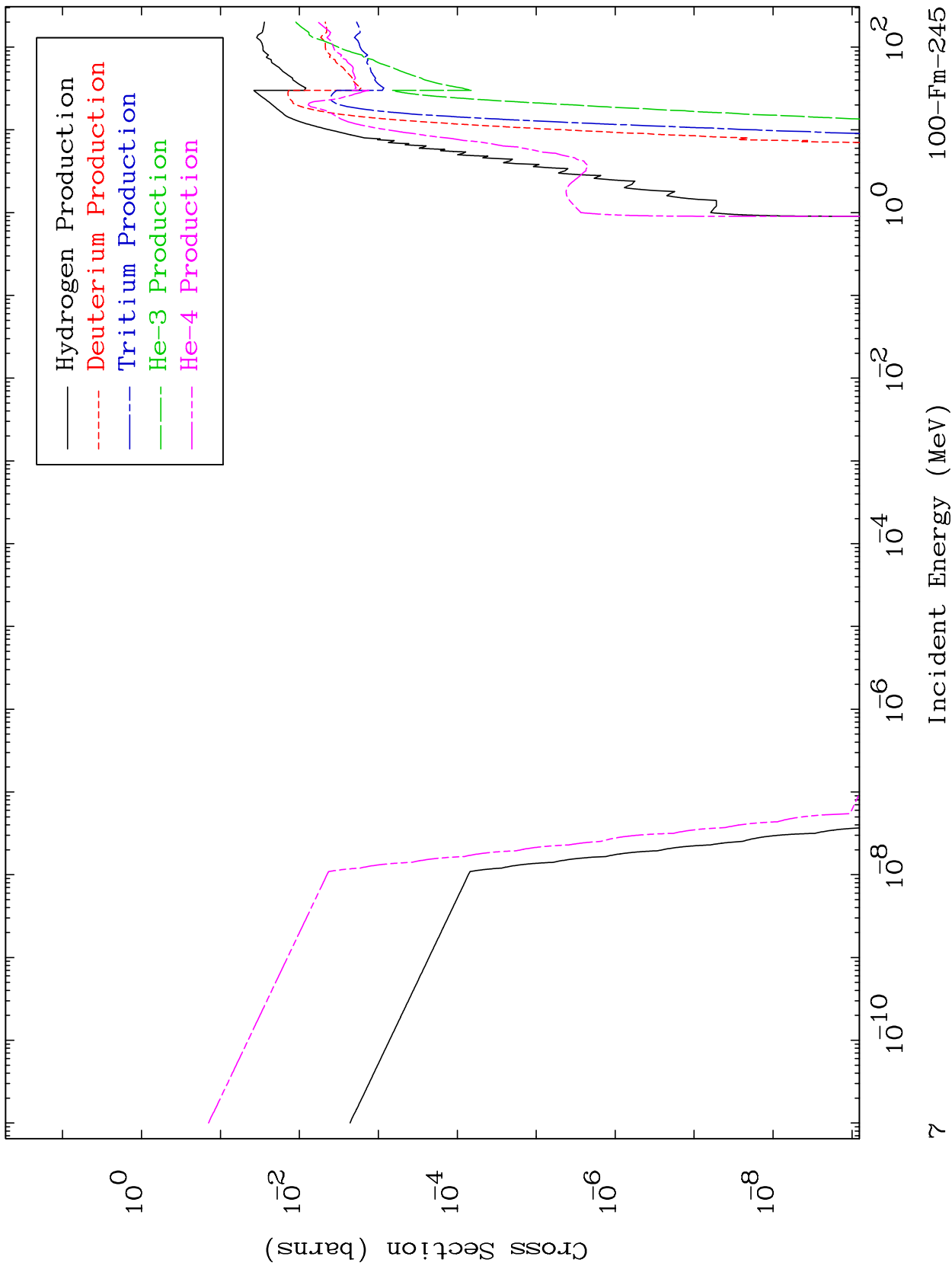


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Charged Particle
293 Kelvin Cross Sections

100-Fm-245

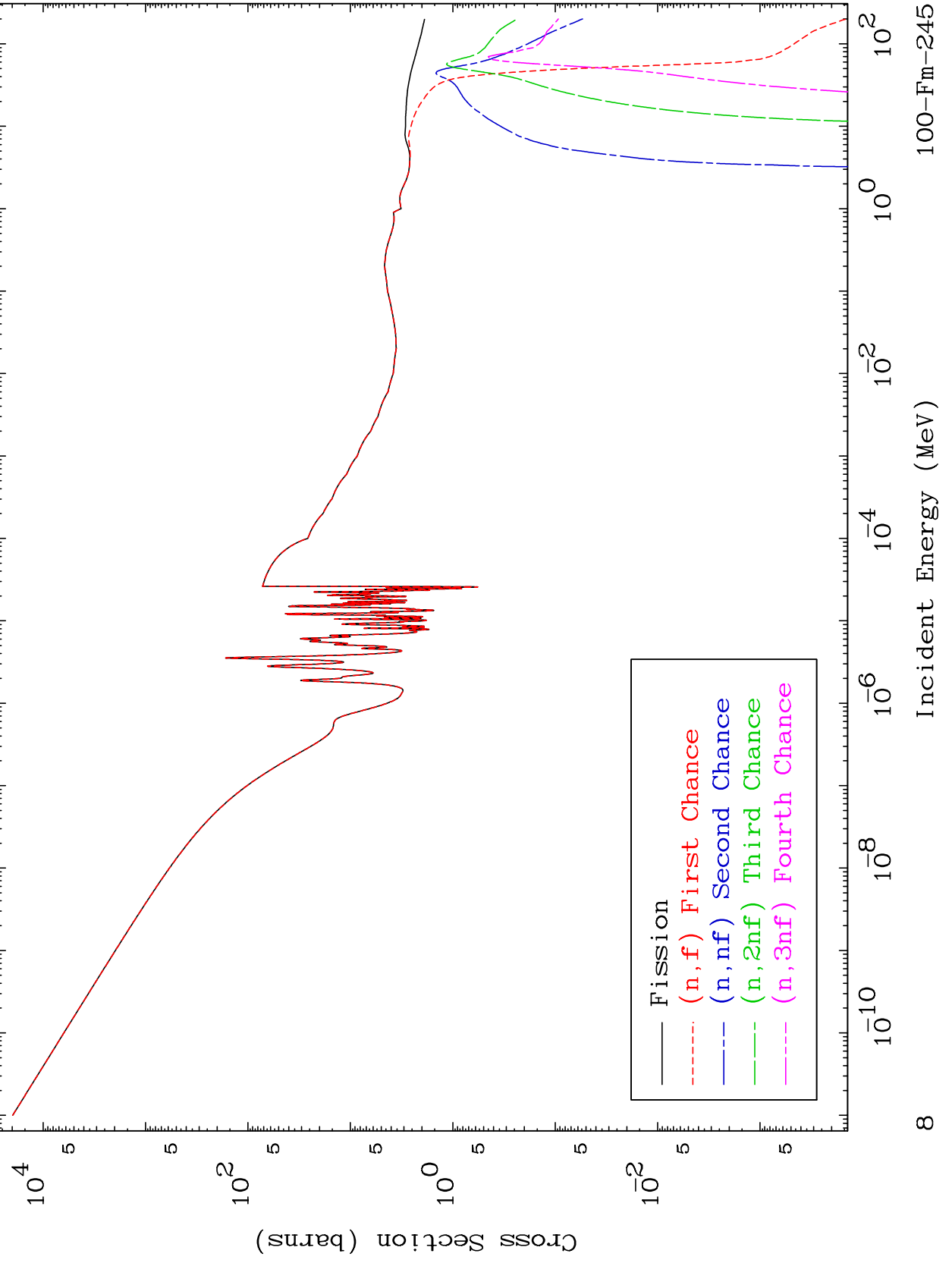




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Fission
293 Kelvin Cross Sections

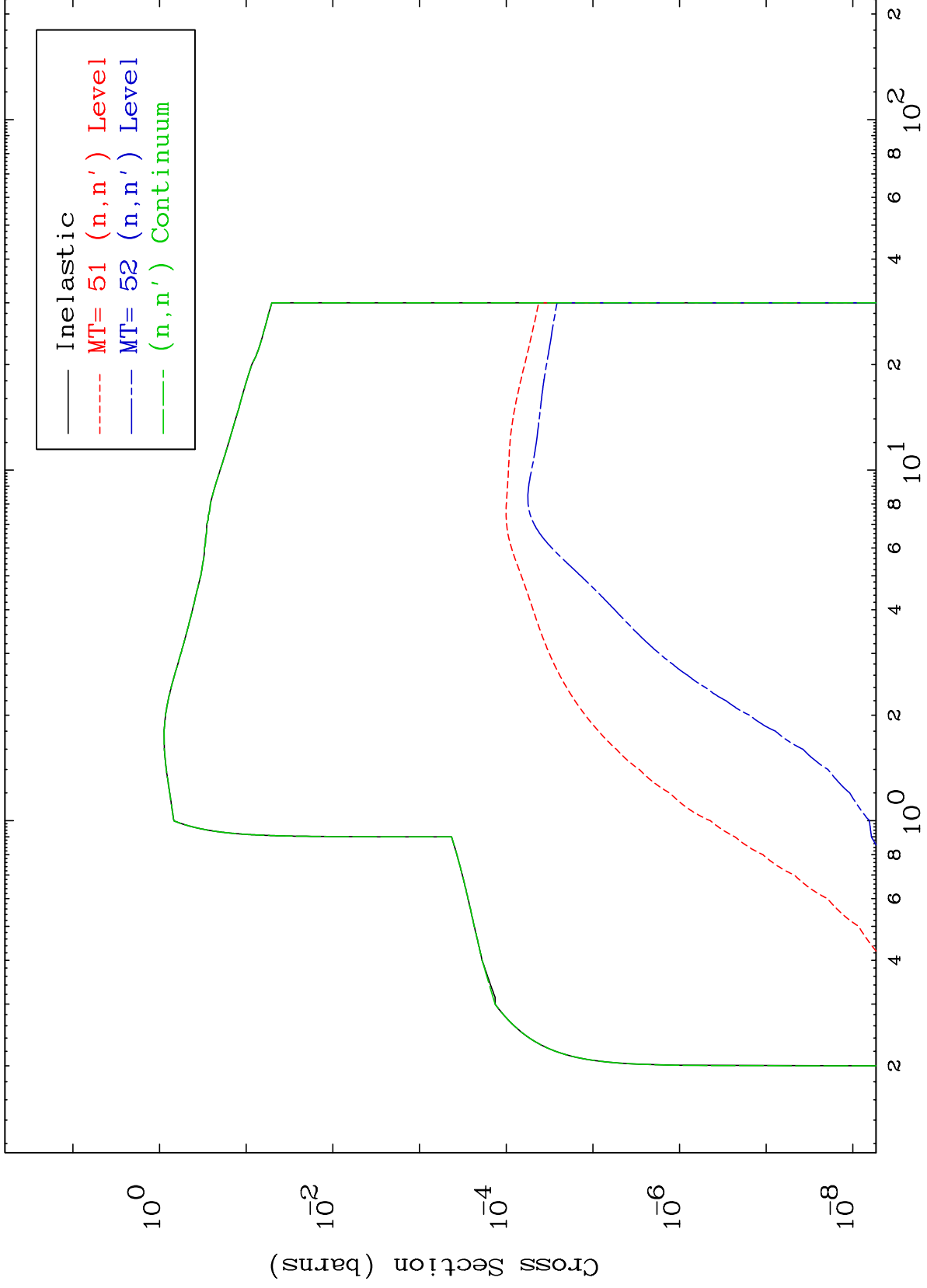
100-Fm-245



MAT 9926

(n,n') Levels
293 Kelvin Cross Sections

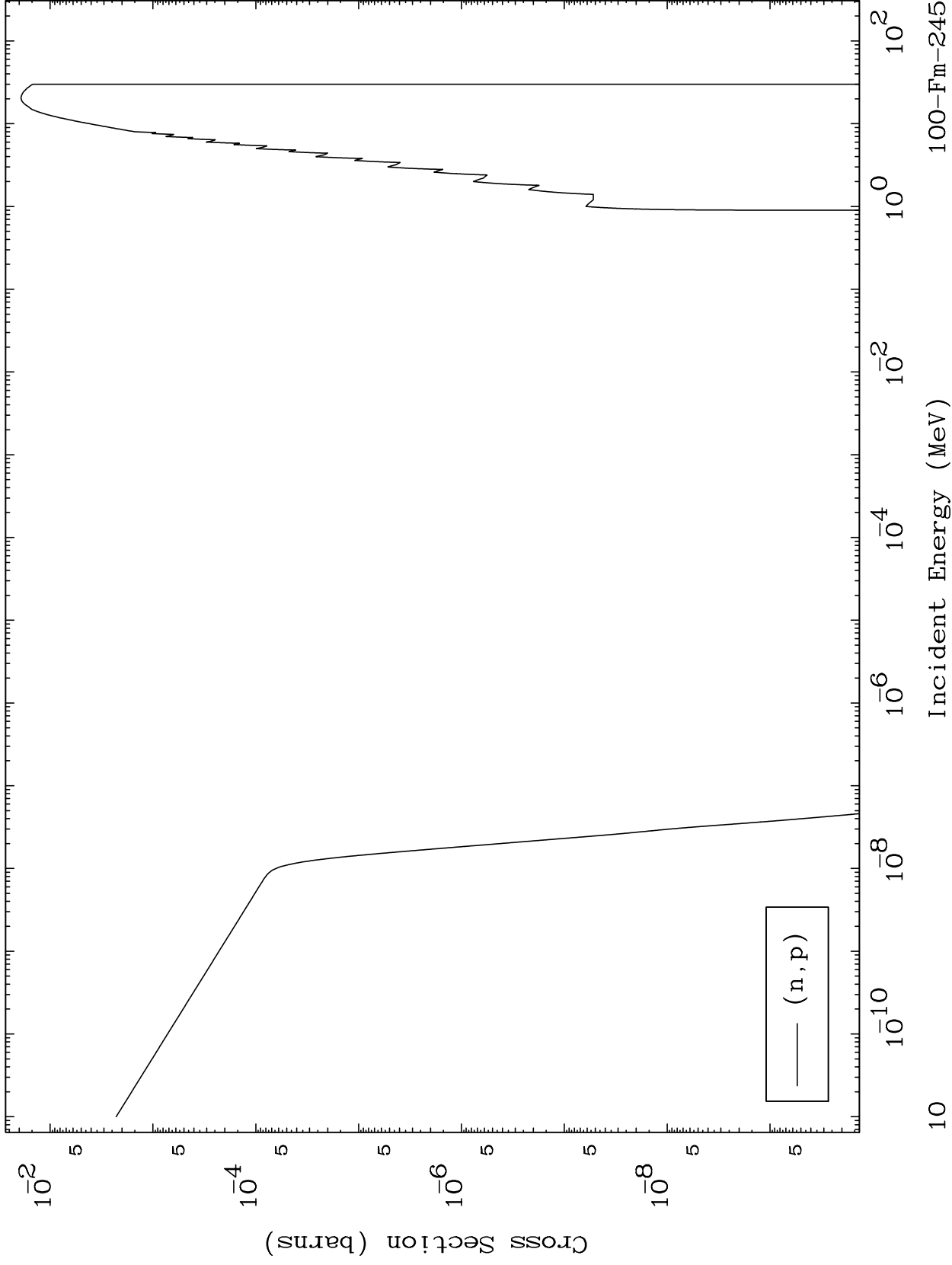
100-Fm-245



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(n,p) Levels
293 Kelvin Cross Sections

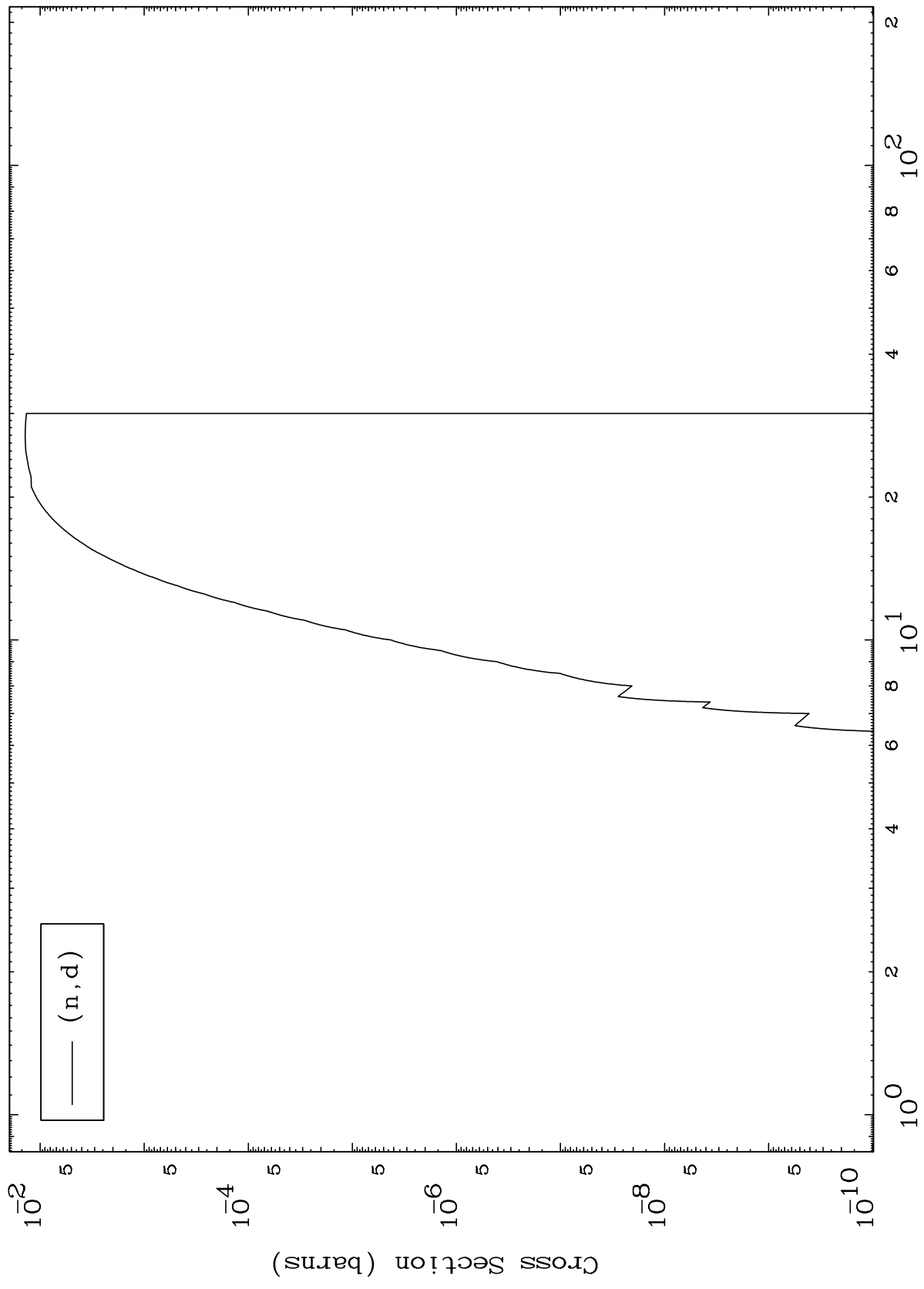
100-Fm-245



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(n,d) Levels
293 Kelvin Cross Sections

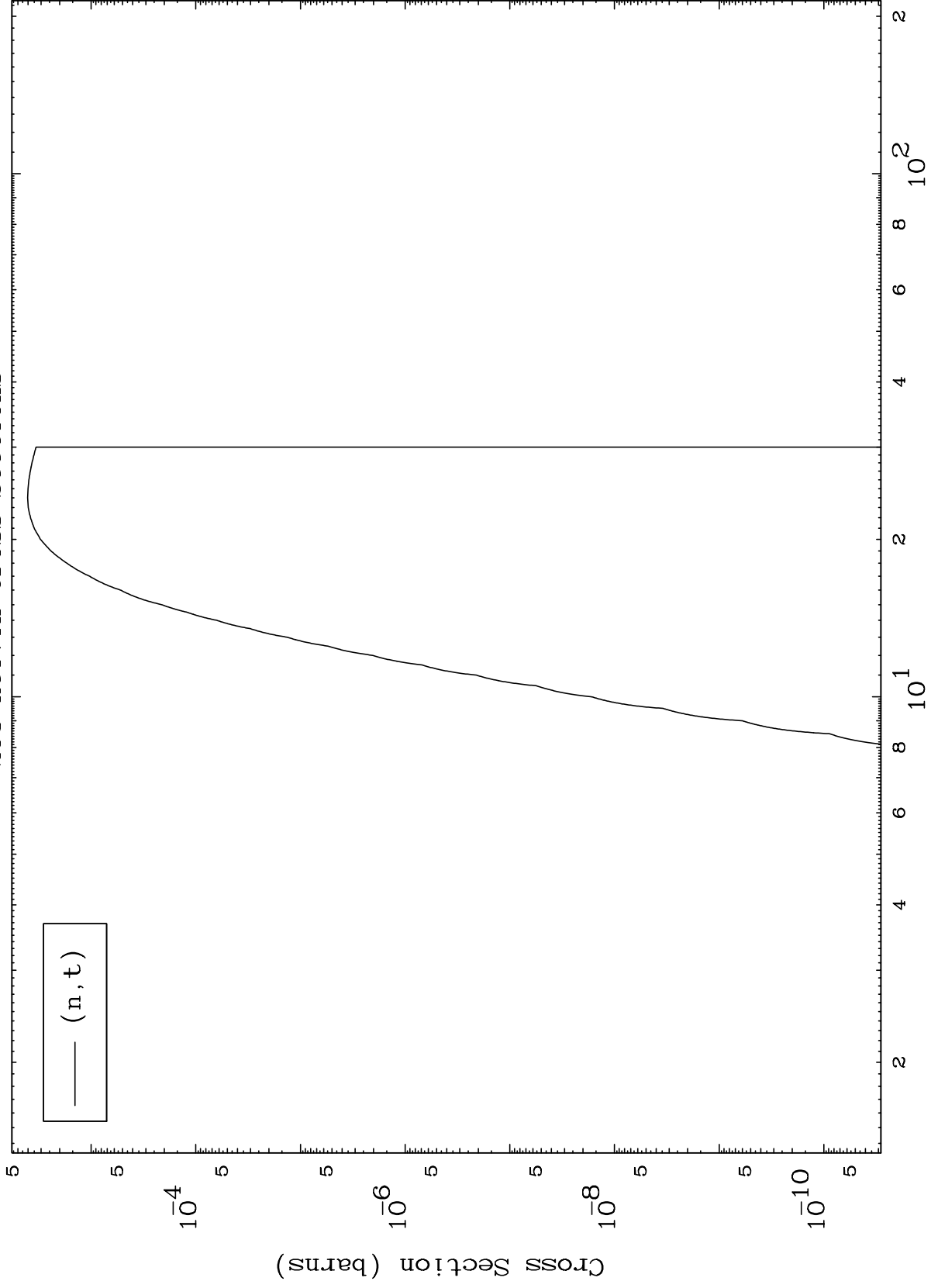
100-Fm-245



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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-245



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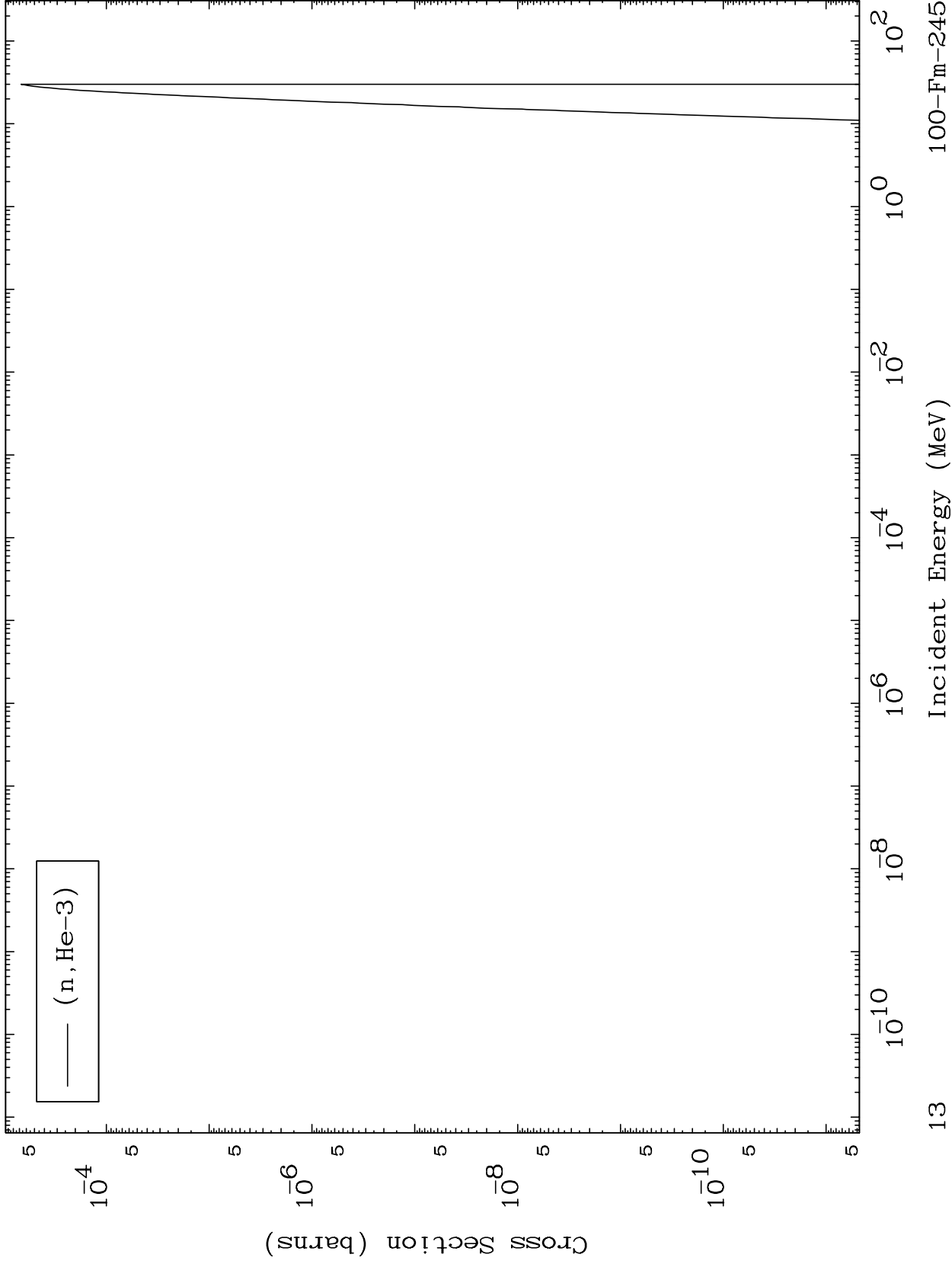
Incident Energy (MeV)

100-Fm-245

MAT 9926

(n,He3) Levels
293 Kelvin Cross Sections

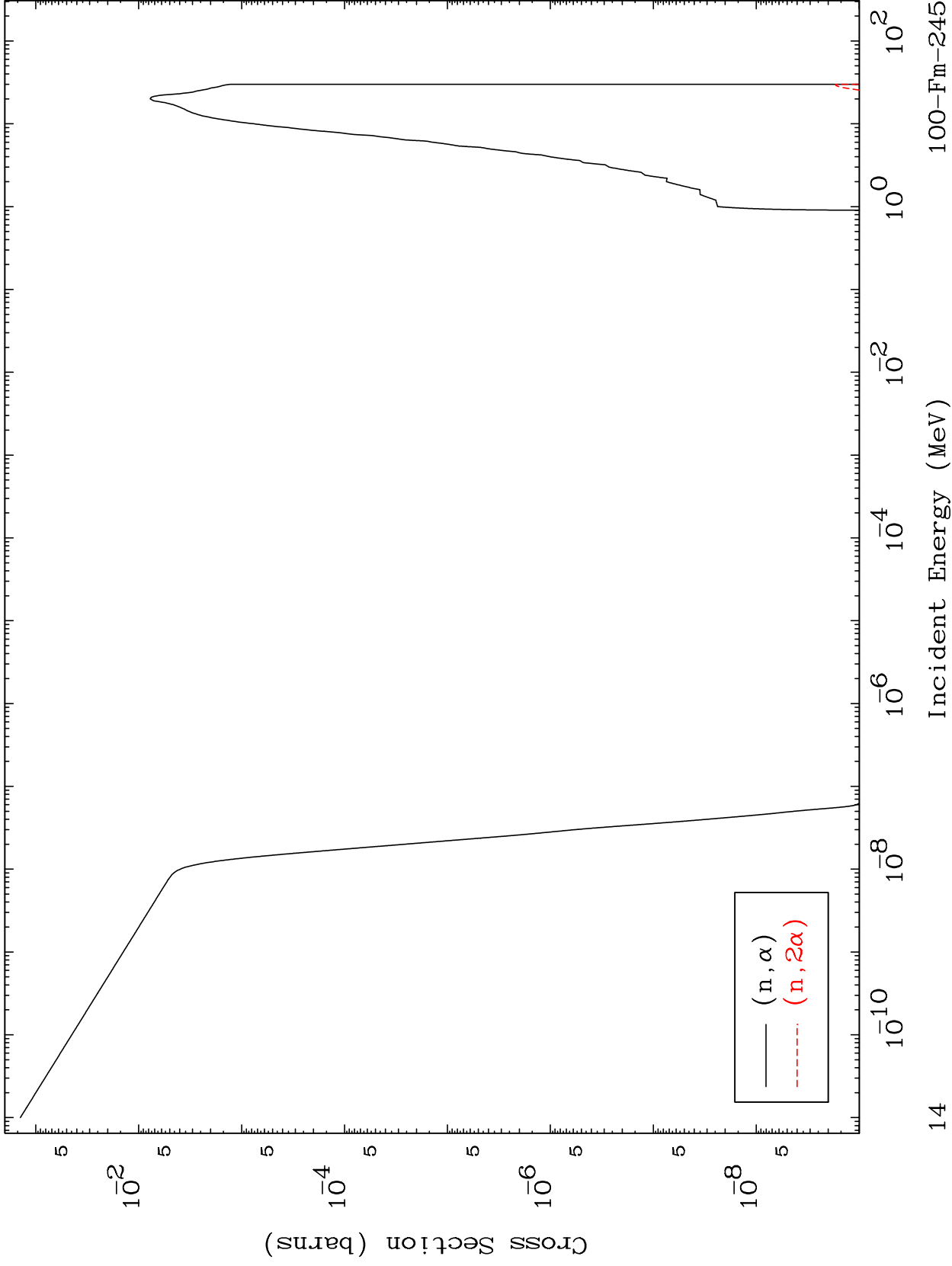
100-Fm-245



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(n, α) Levels
293 Kelvin Cross Sections

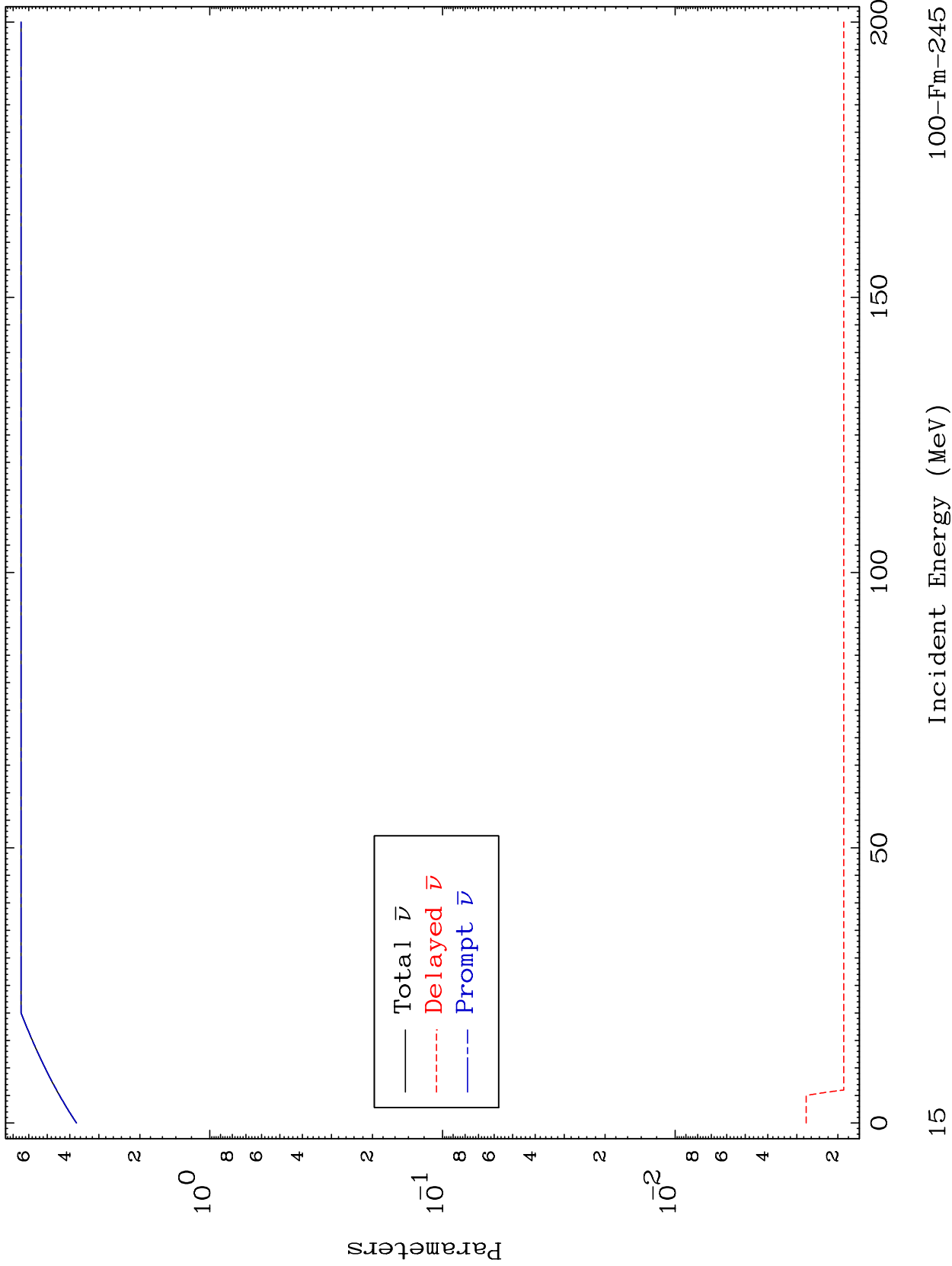
100-Fm-245



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Energy Release
Parameters

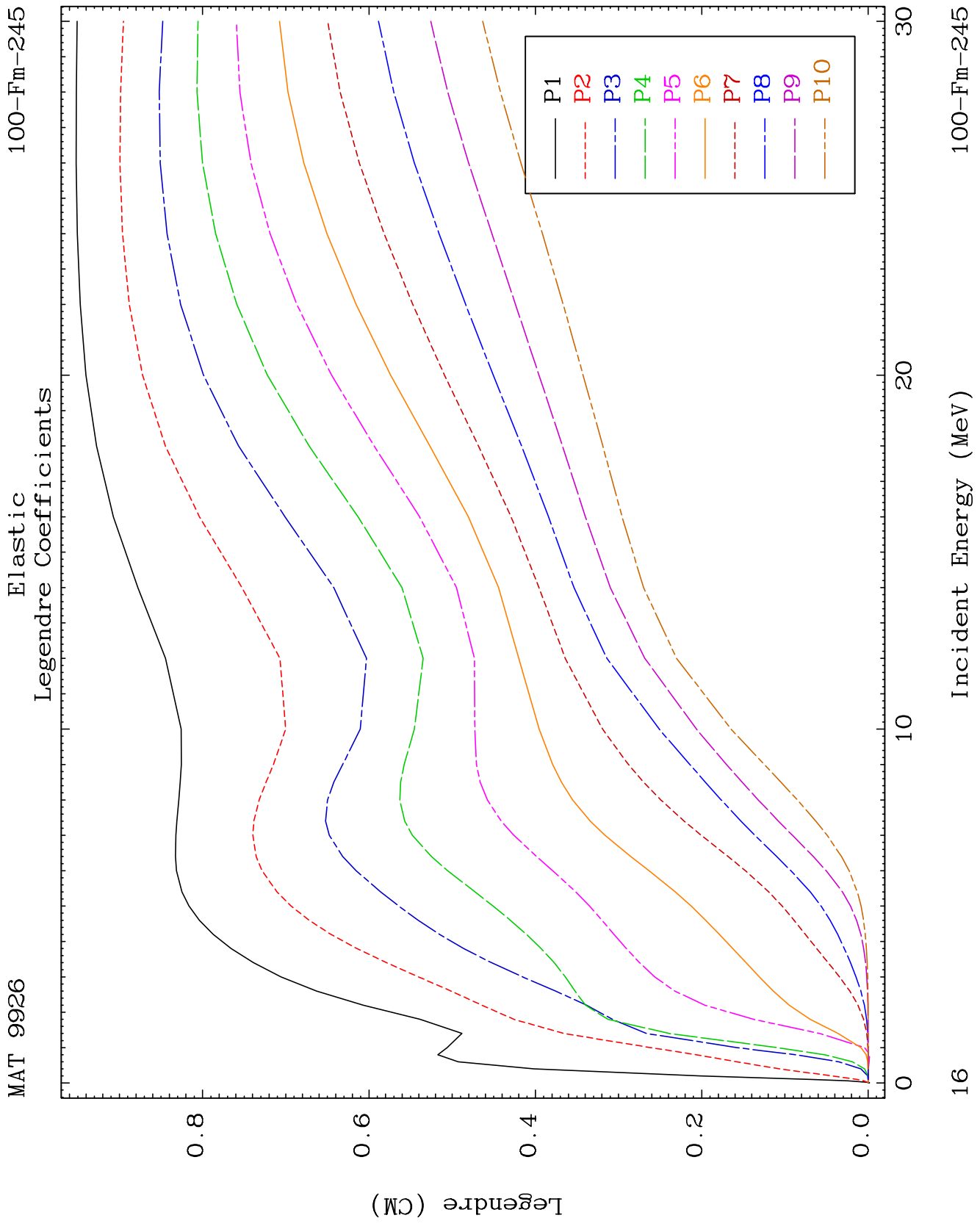
100-Fm-245



15

Incident Energy (MeV)

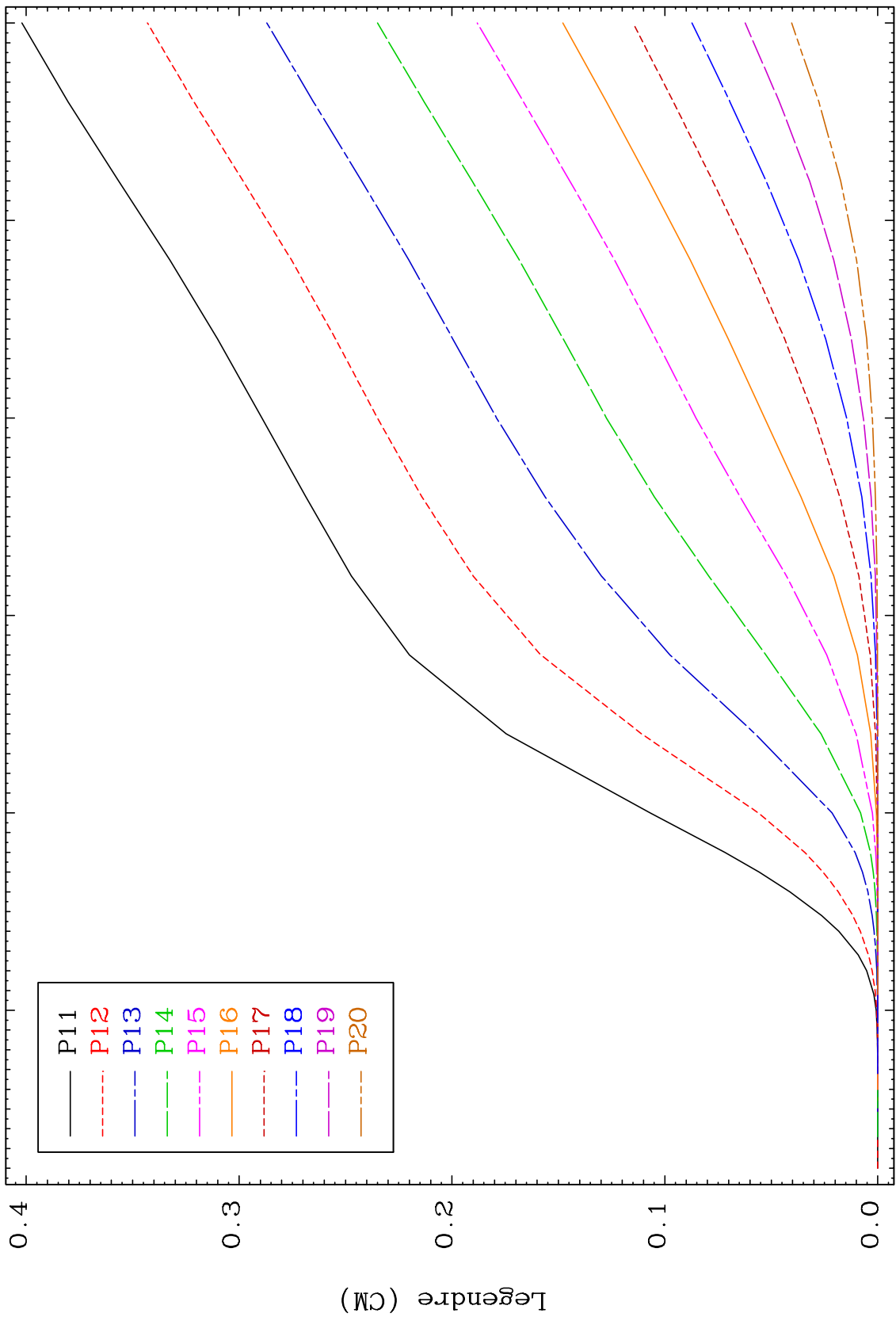
100-Fm-245



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Elastic Legendre Coefficients

100-Fm-245



17

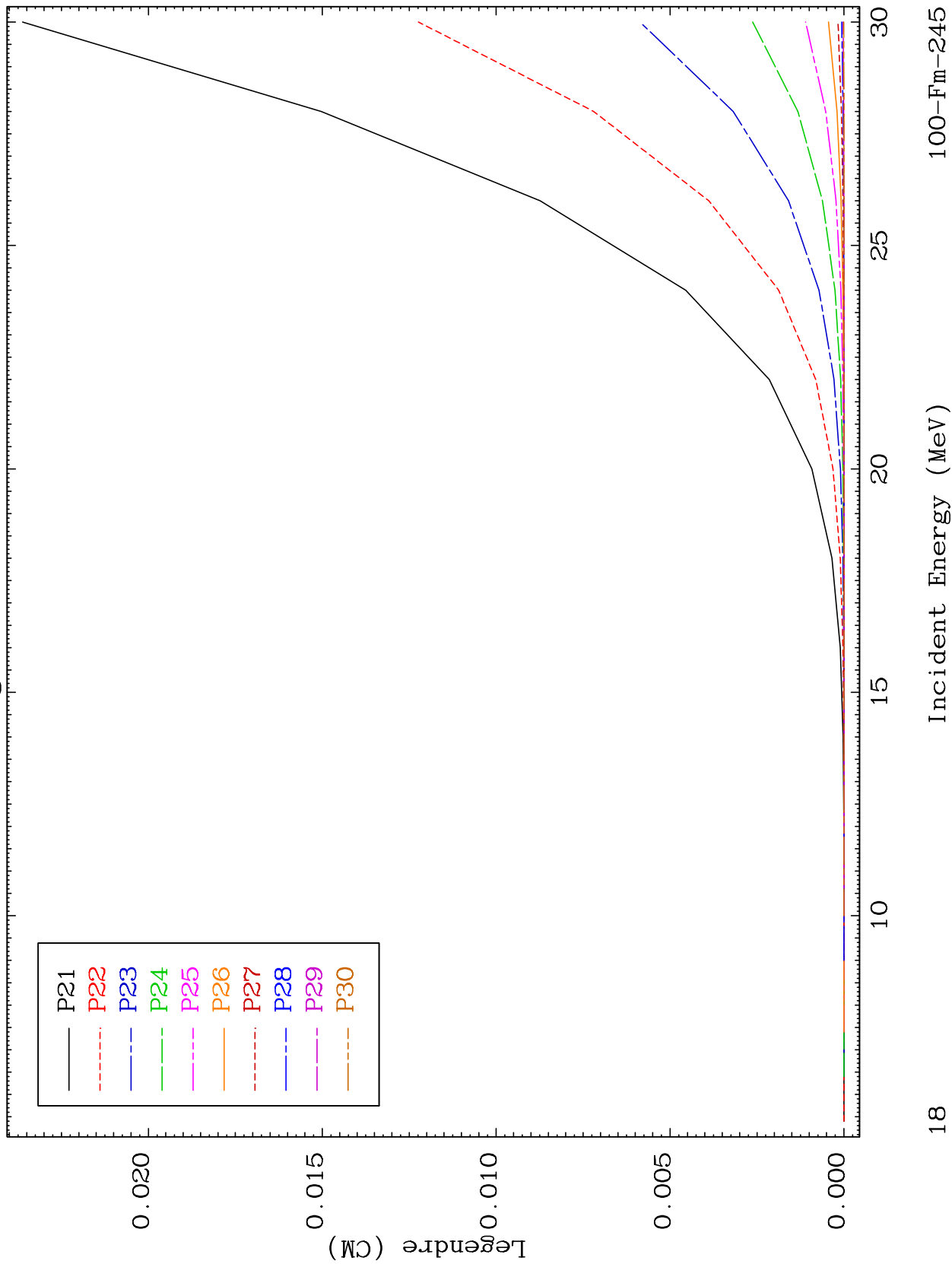
Incident Energy (MeV)

100-Fm-245

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Elastic Legendre Coefficients

100-Fm-245



18

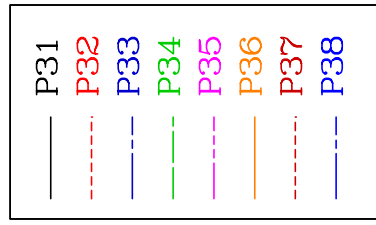
Incident Energy (MeV)

100-Fm-245

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Elastic
Legendre Coefficients

100-Fm-245



$\times 10^{-6}$

2.0

1.5

1.0

0.5

0.0

Legendre (CM)

15

20

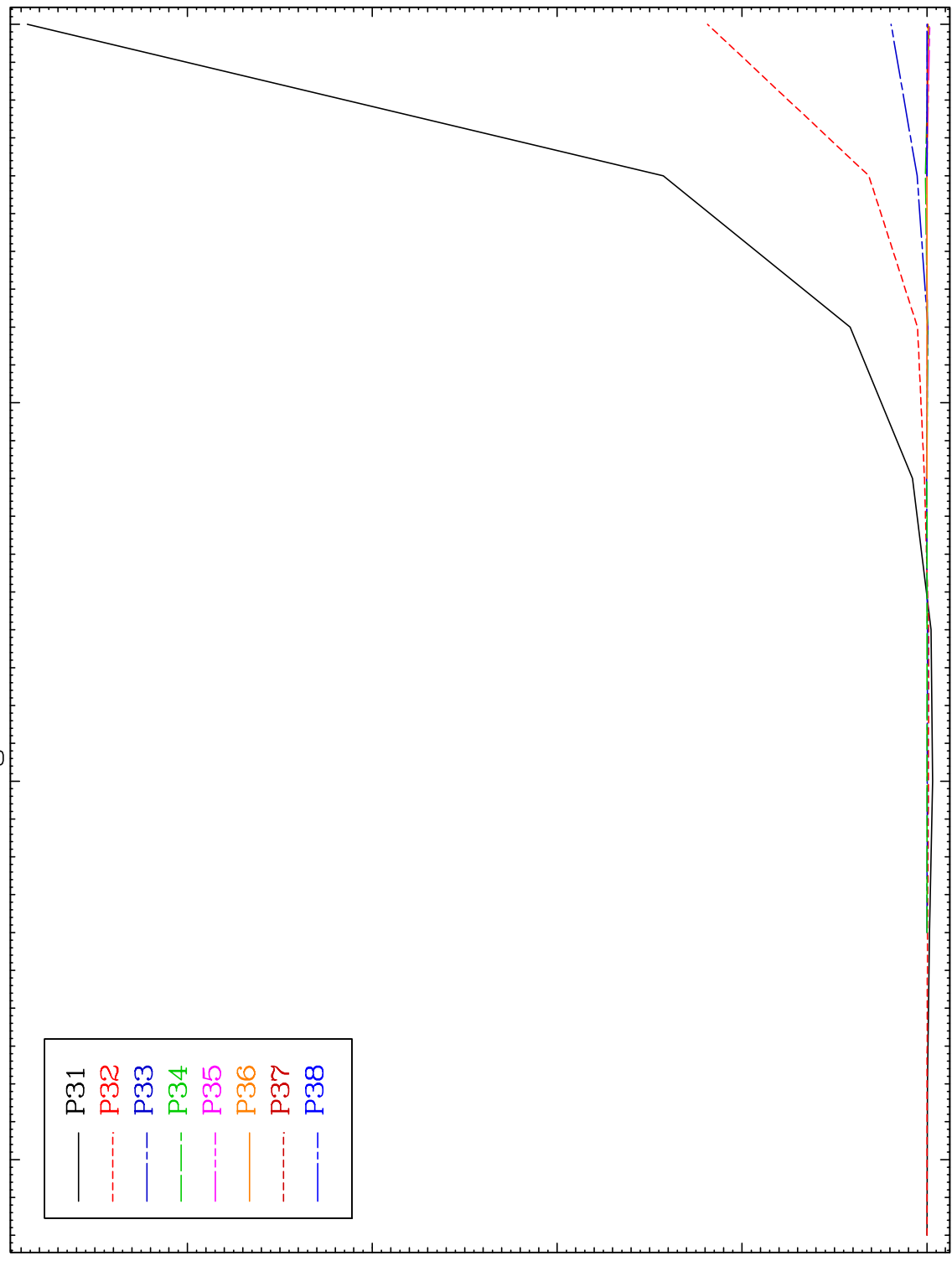
25

30

19

Incident Energy (MeV)

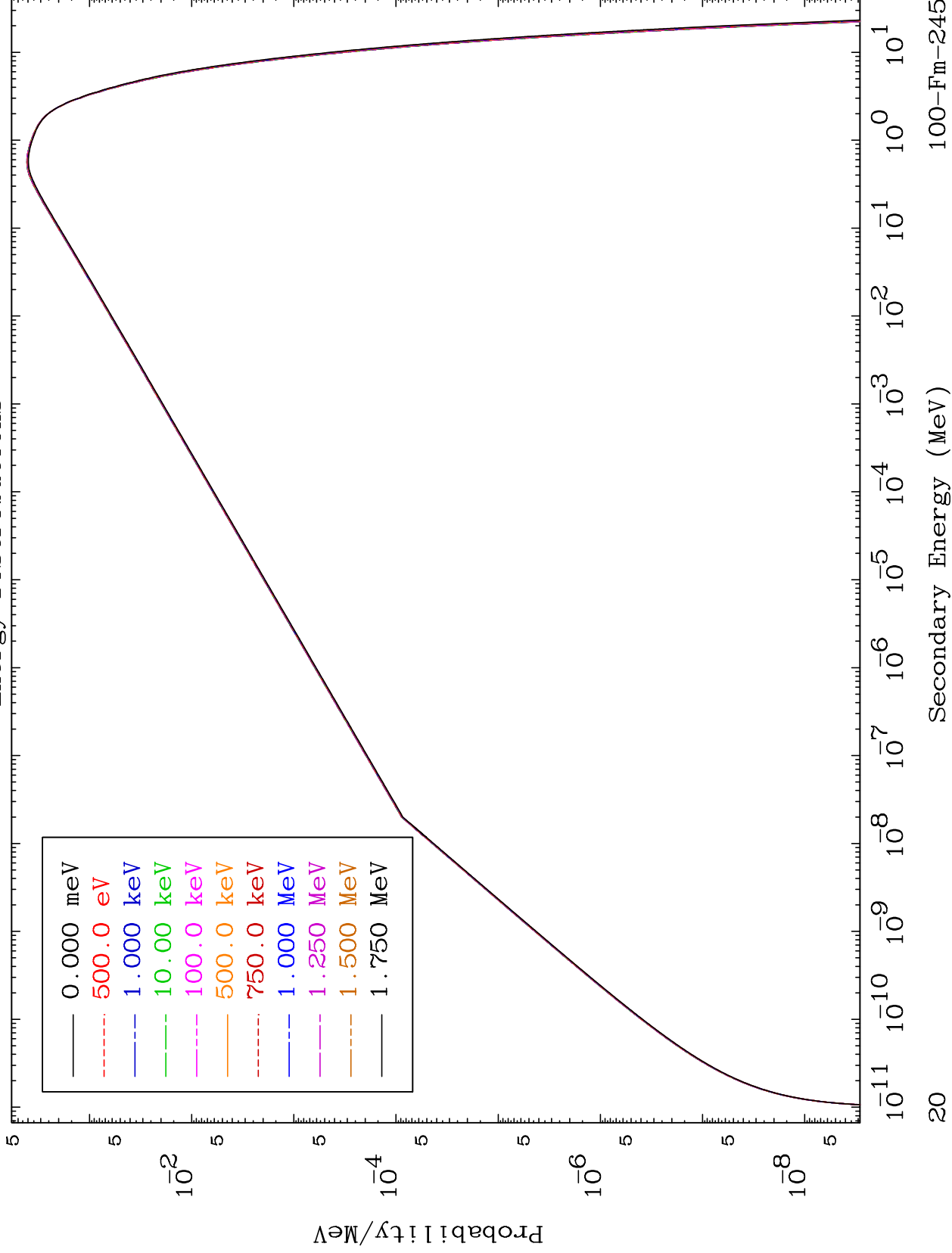
100-Fm-245



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Fission Energy Distributions

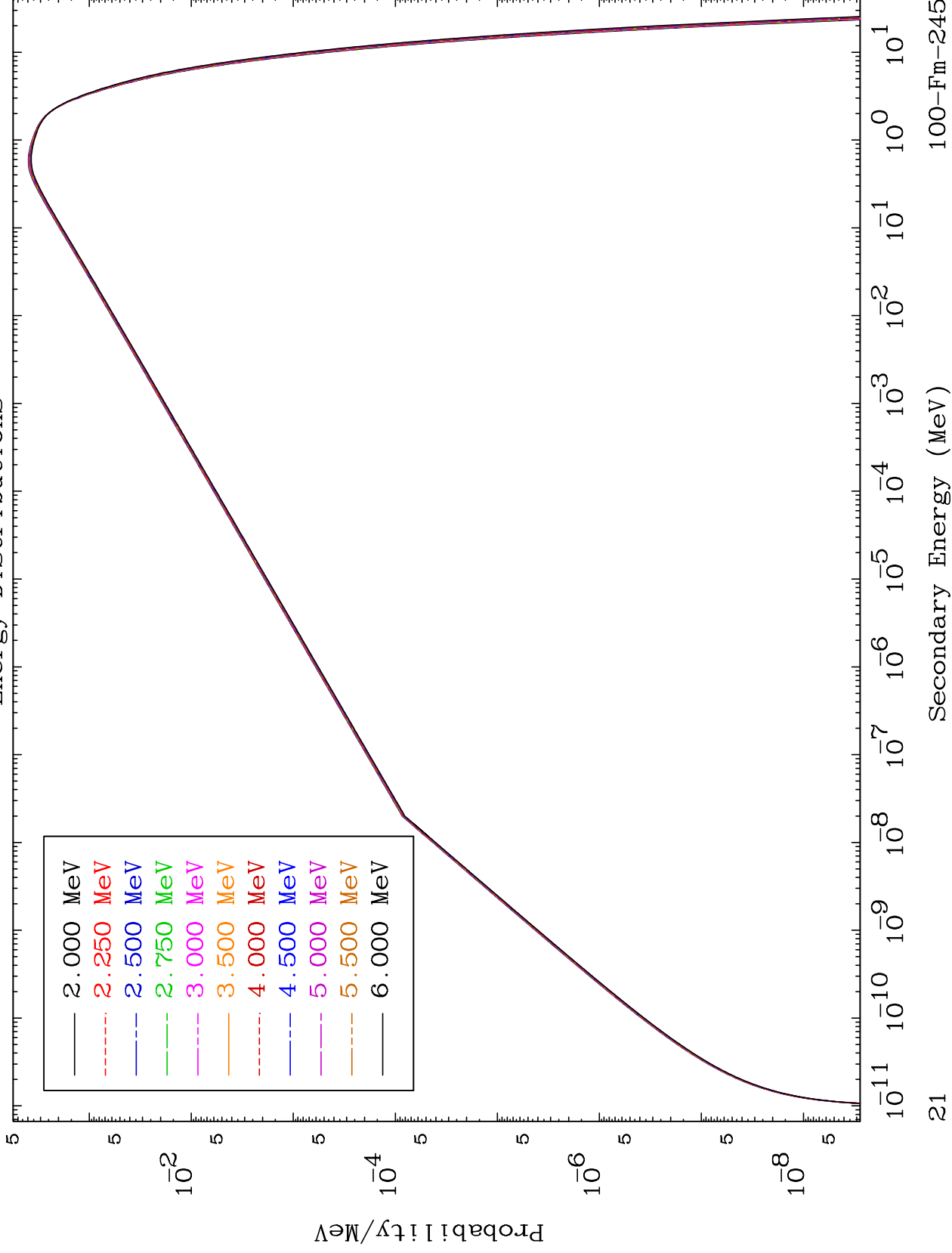
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Fission Energy Distributions

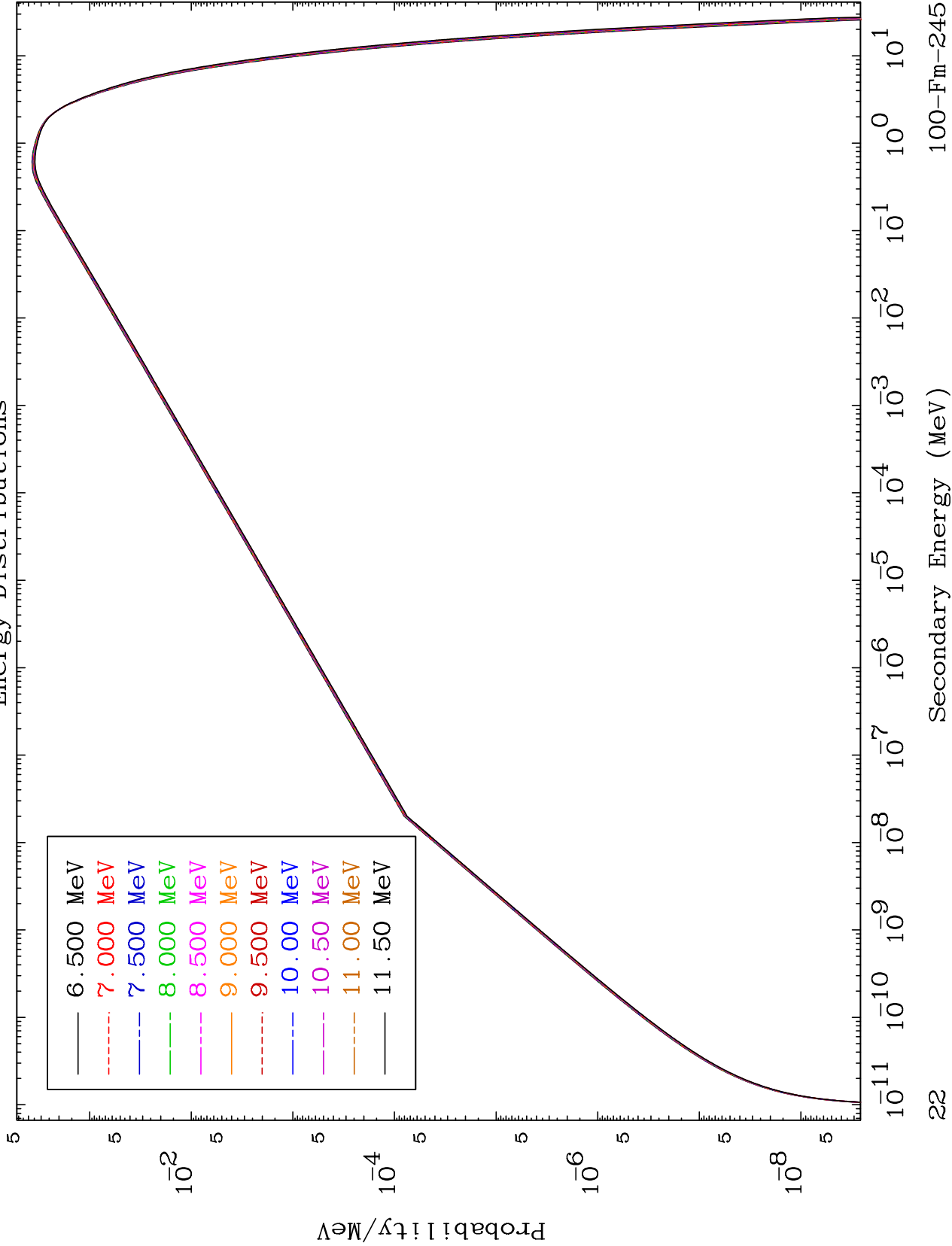
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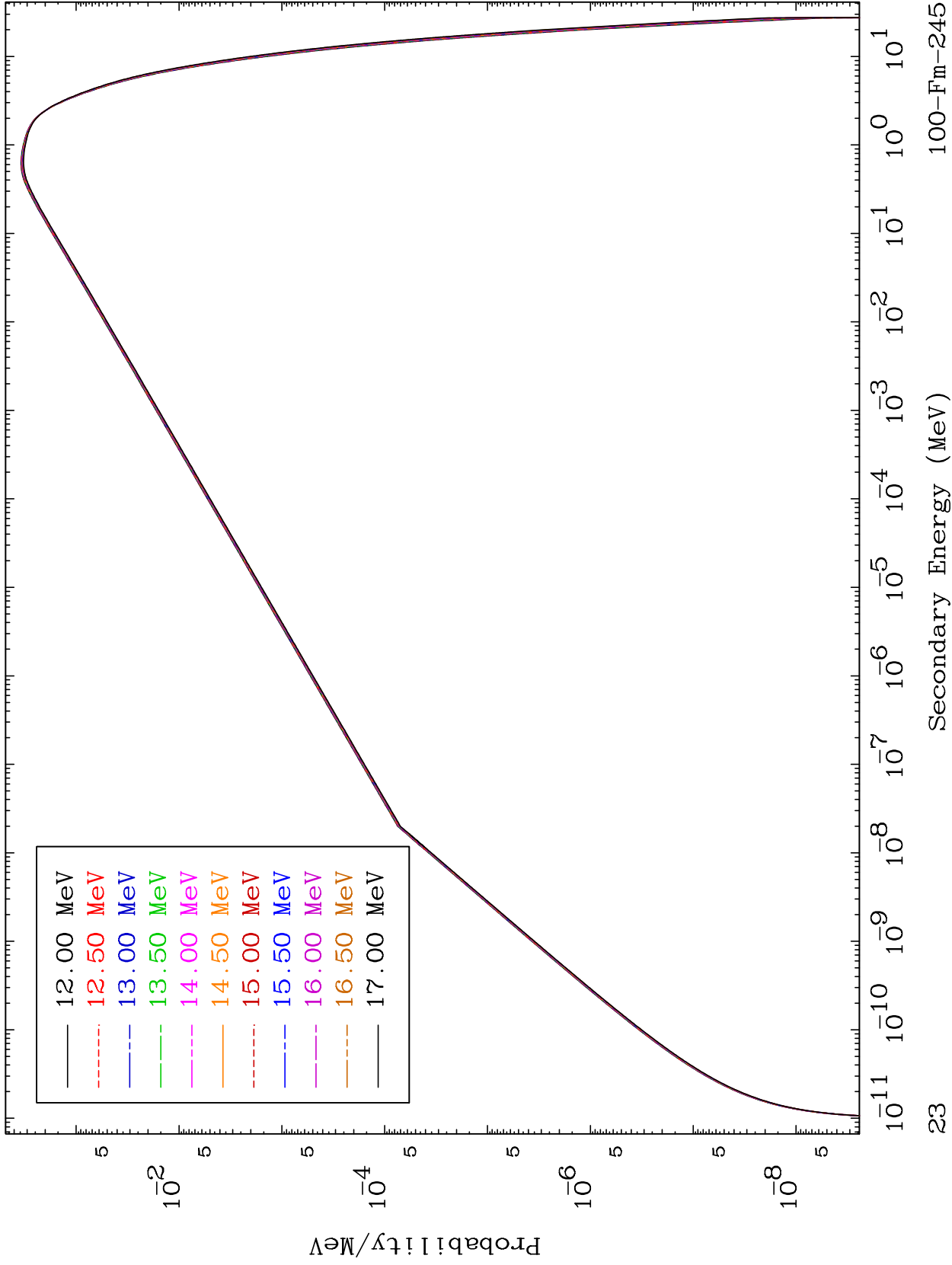
Fission Energy Distributions

100-Fm-245



100-Fm-245

Fission Energy Distributions



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Fission Energy Distributions

100-Fm-245

