

Program EVALPLOT
(Version 2021-1)

by

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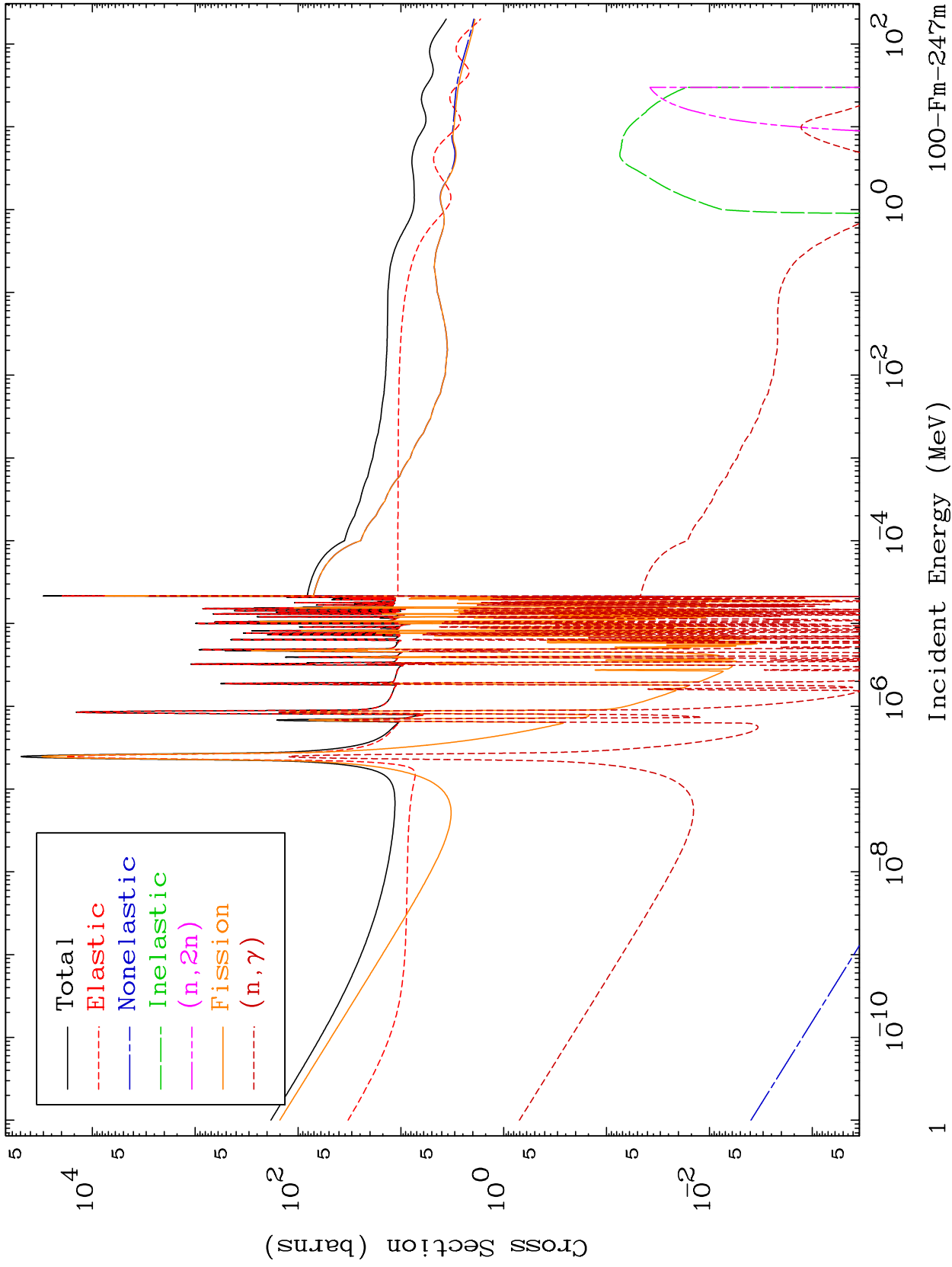
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9933

Neutron Major
293 Kelvin Cross Sections

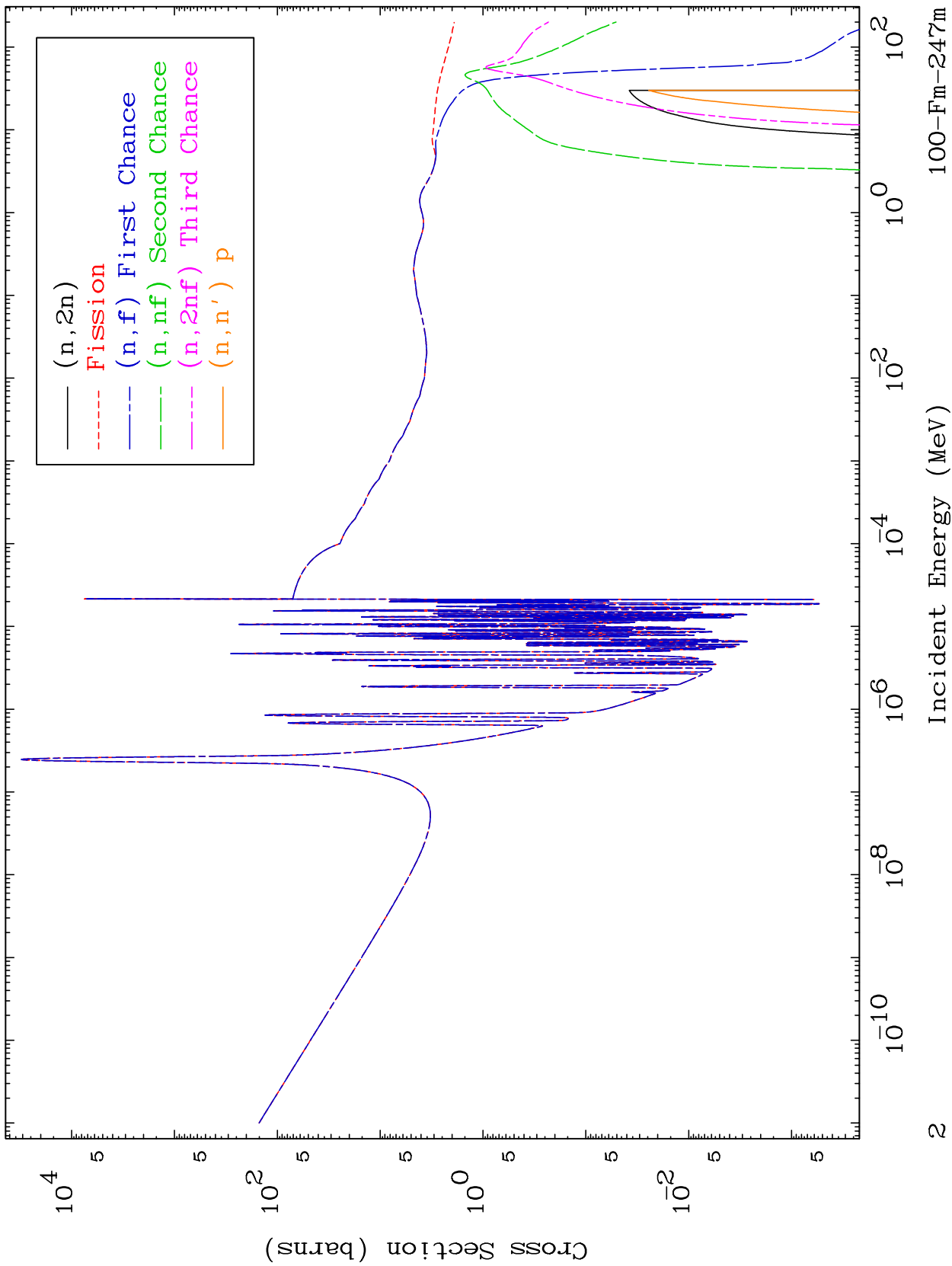
100-Fm-247m



MAT 9933

Neutron Absorption
293 Kelvin Cross Sections

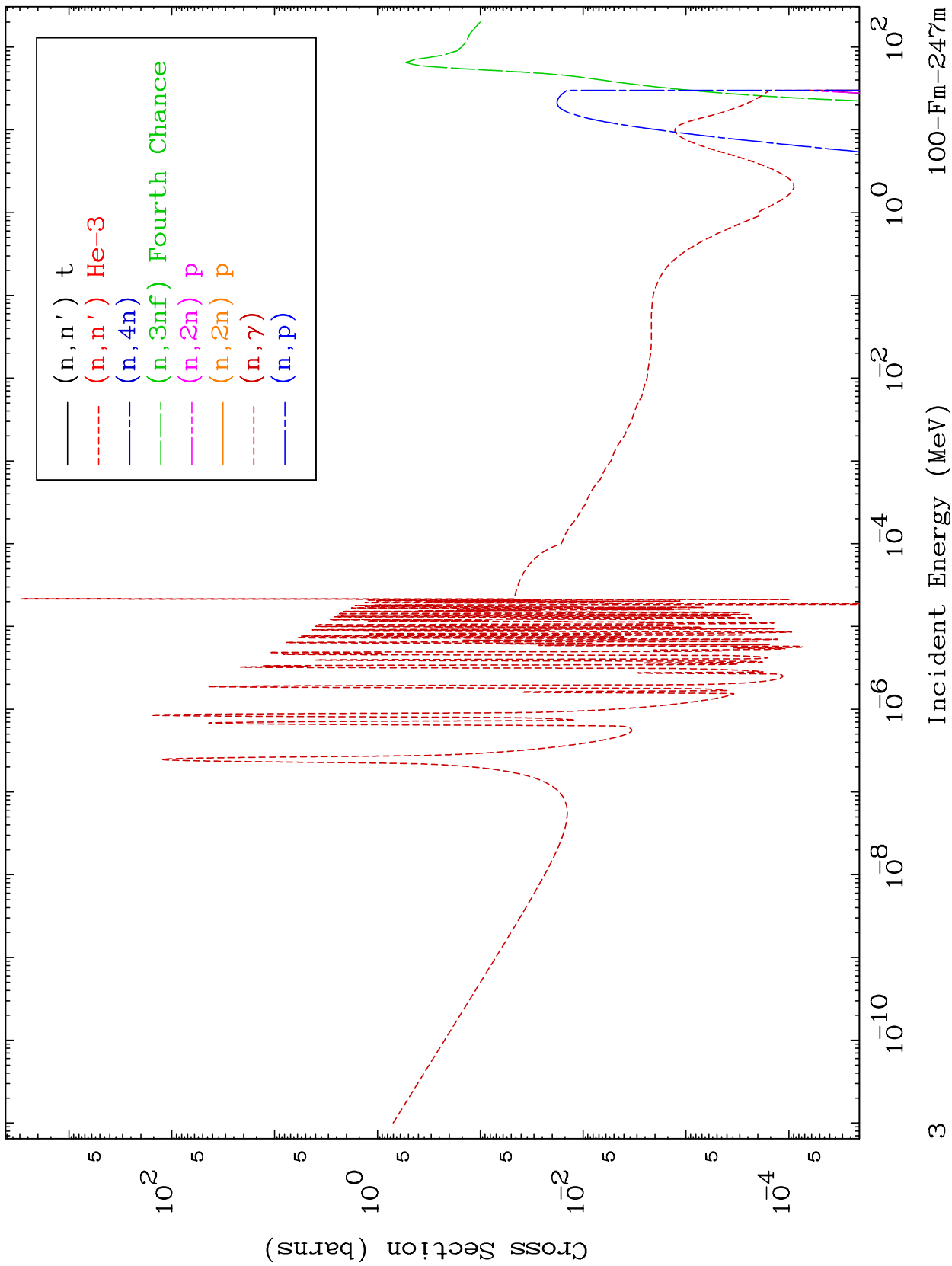
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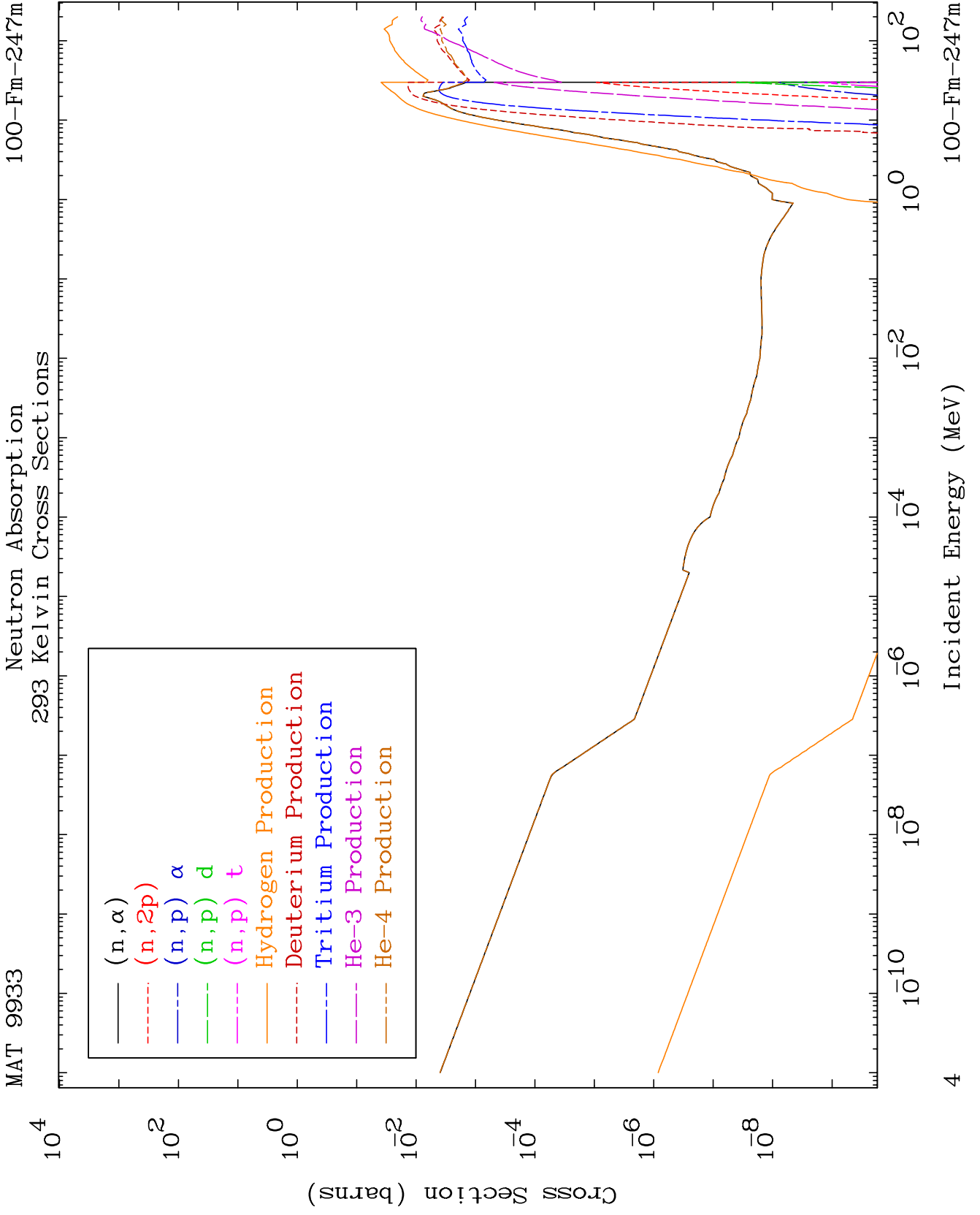


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Neutron Absorption
293 Kelvin Cross Sections

100-Fm-247m

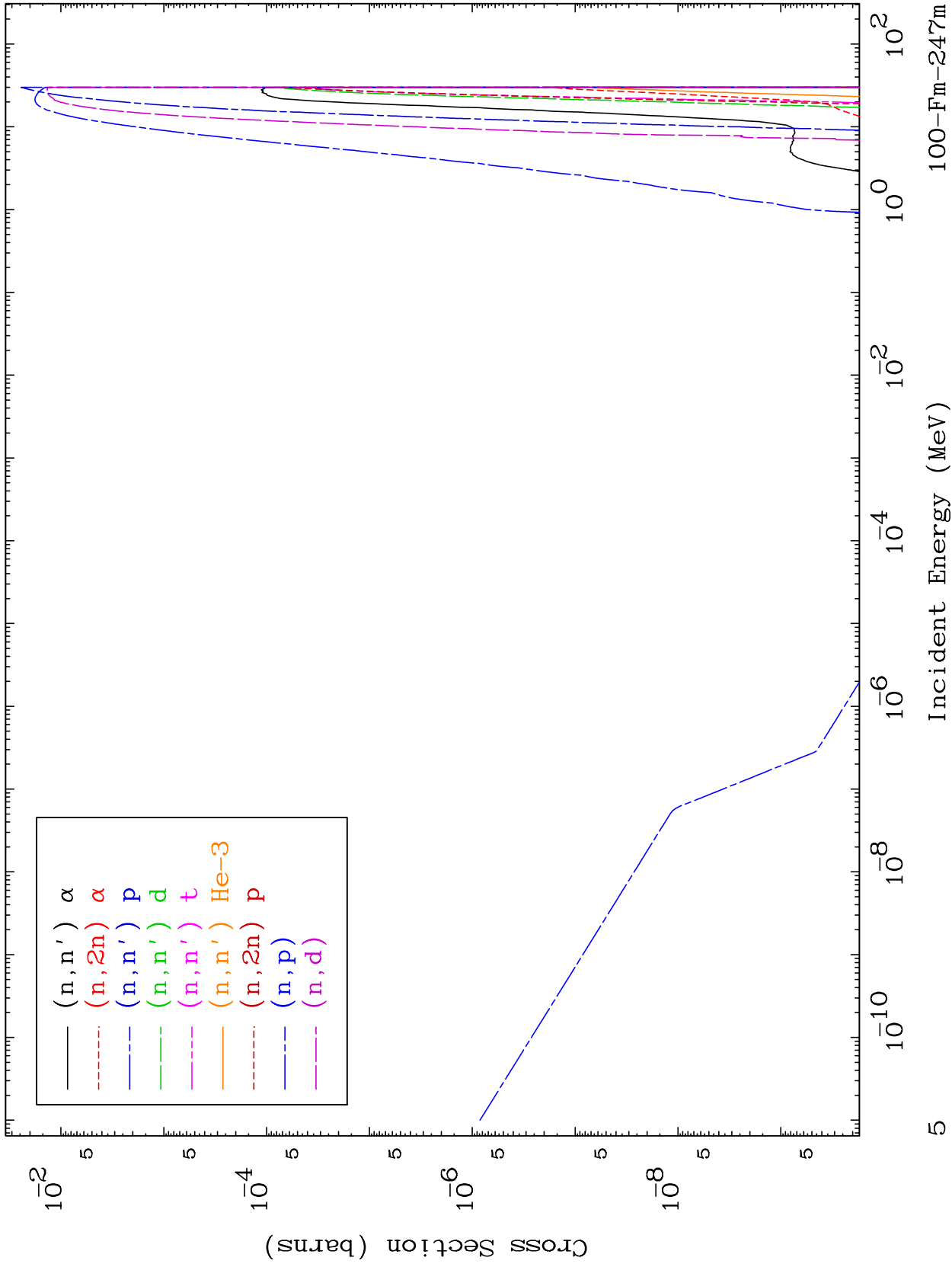


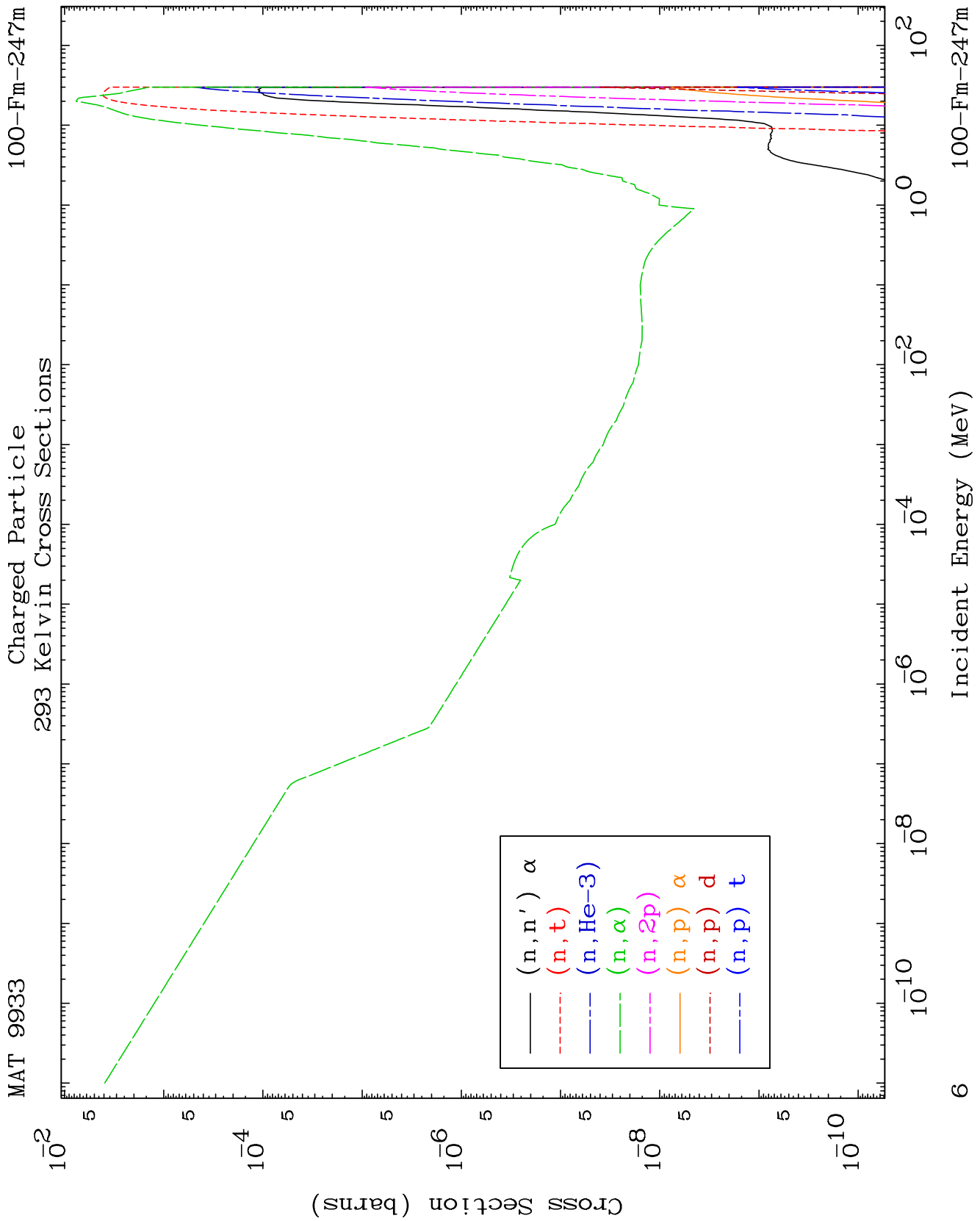


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Charged Particle
293 Kelvin Cross Sections

100-Fm-247m

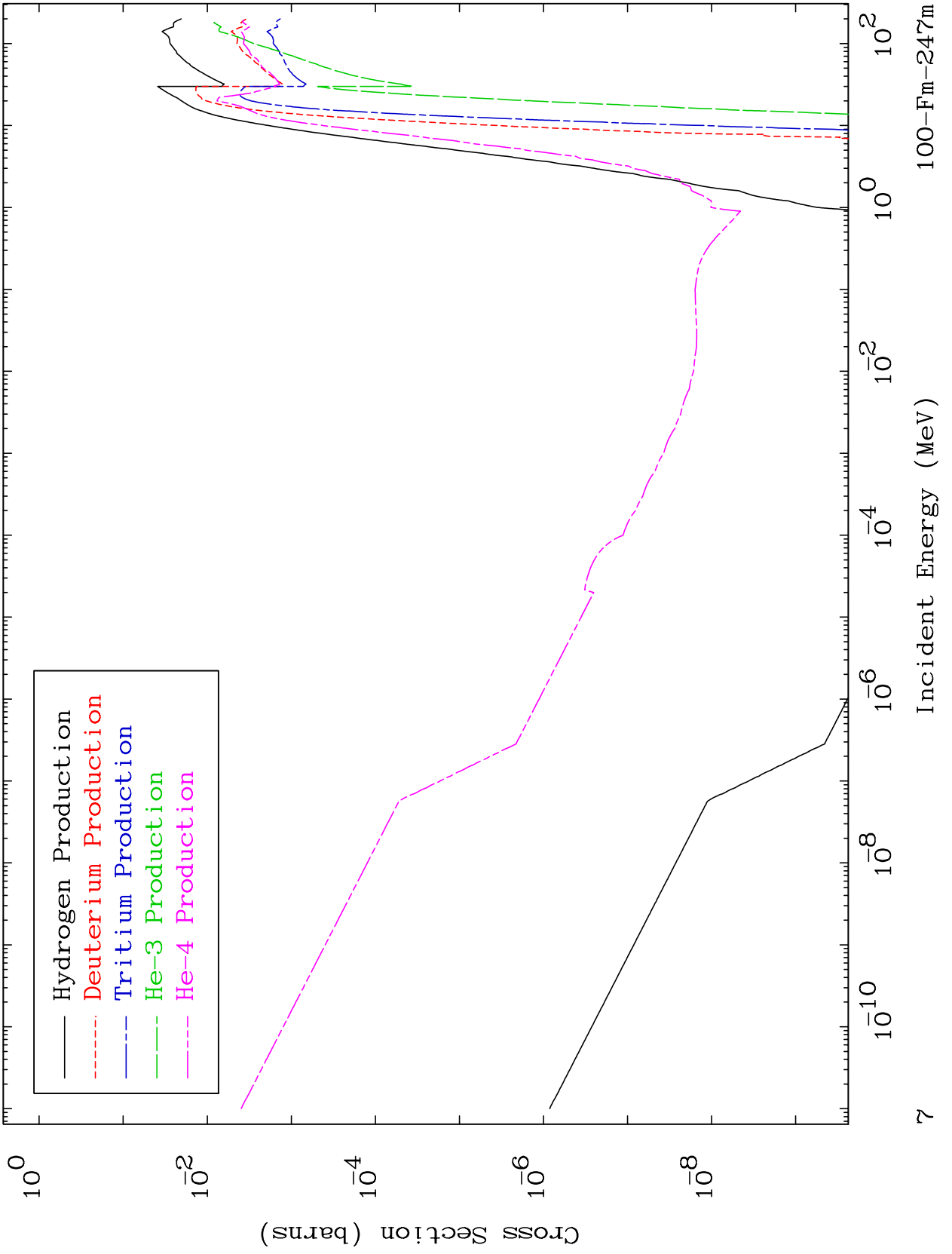




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Particle Production
293 Kelvin Cross Sections

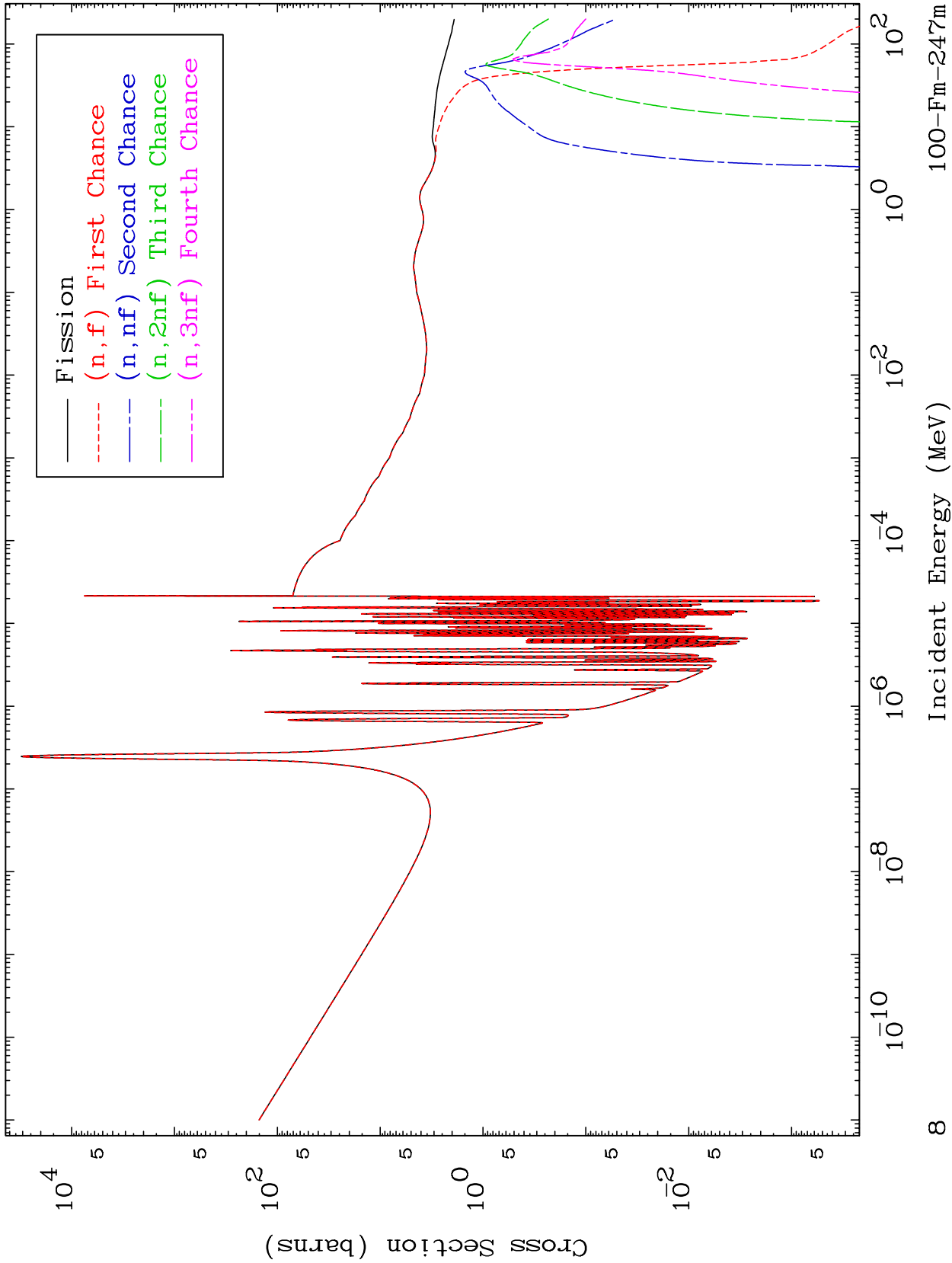
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Fission
293 Kelvin Cross Sections

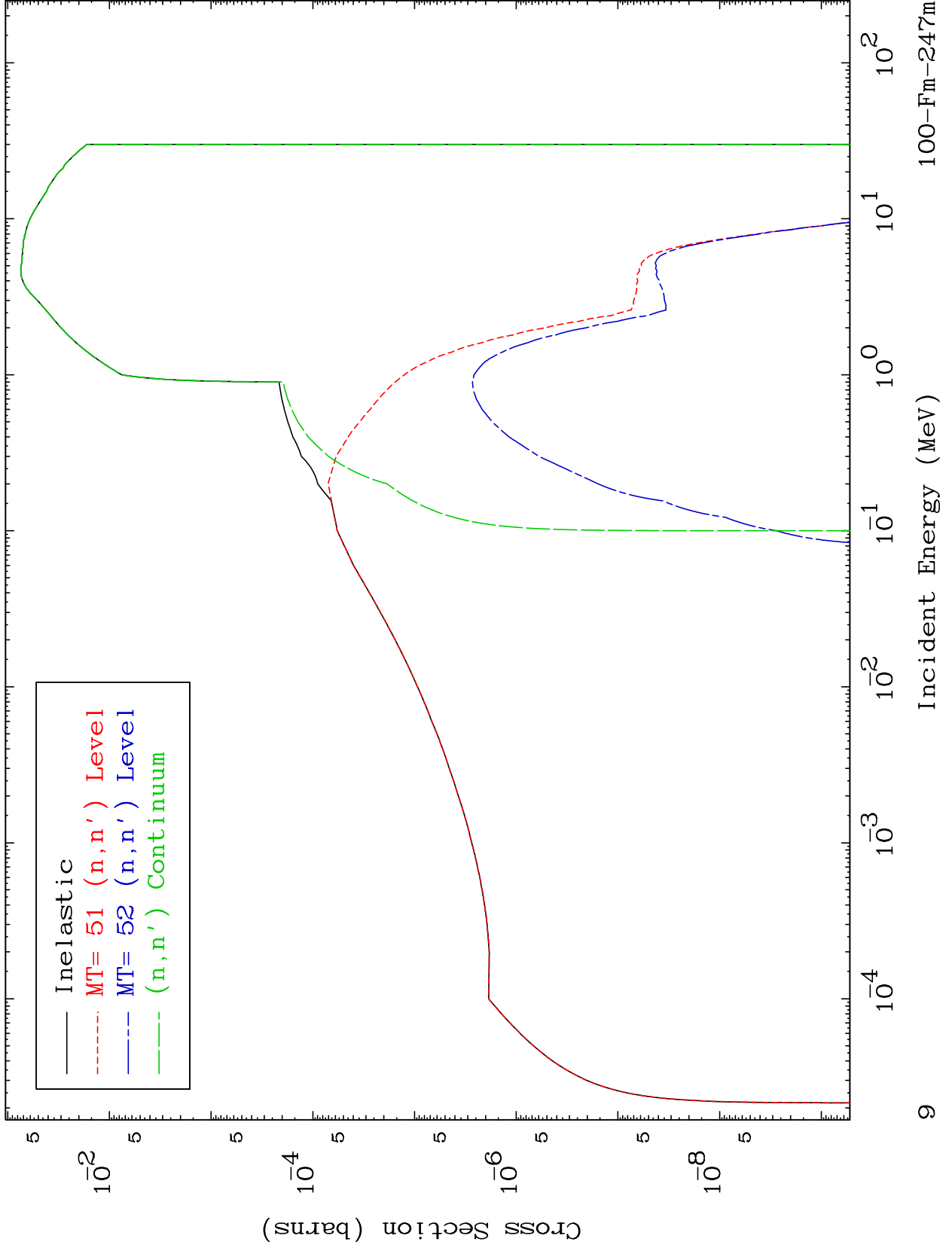
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(n,n') Levels
293 Kelvin Cross Sections

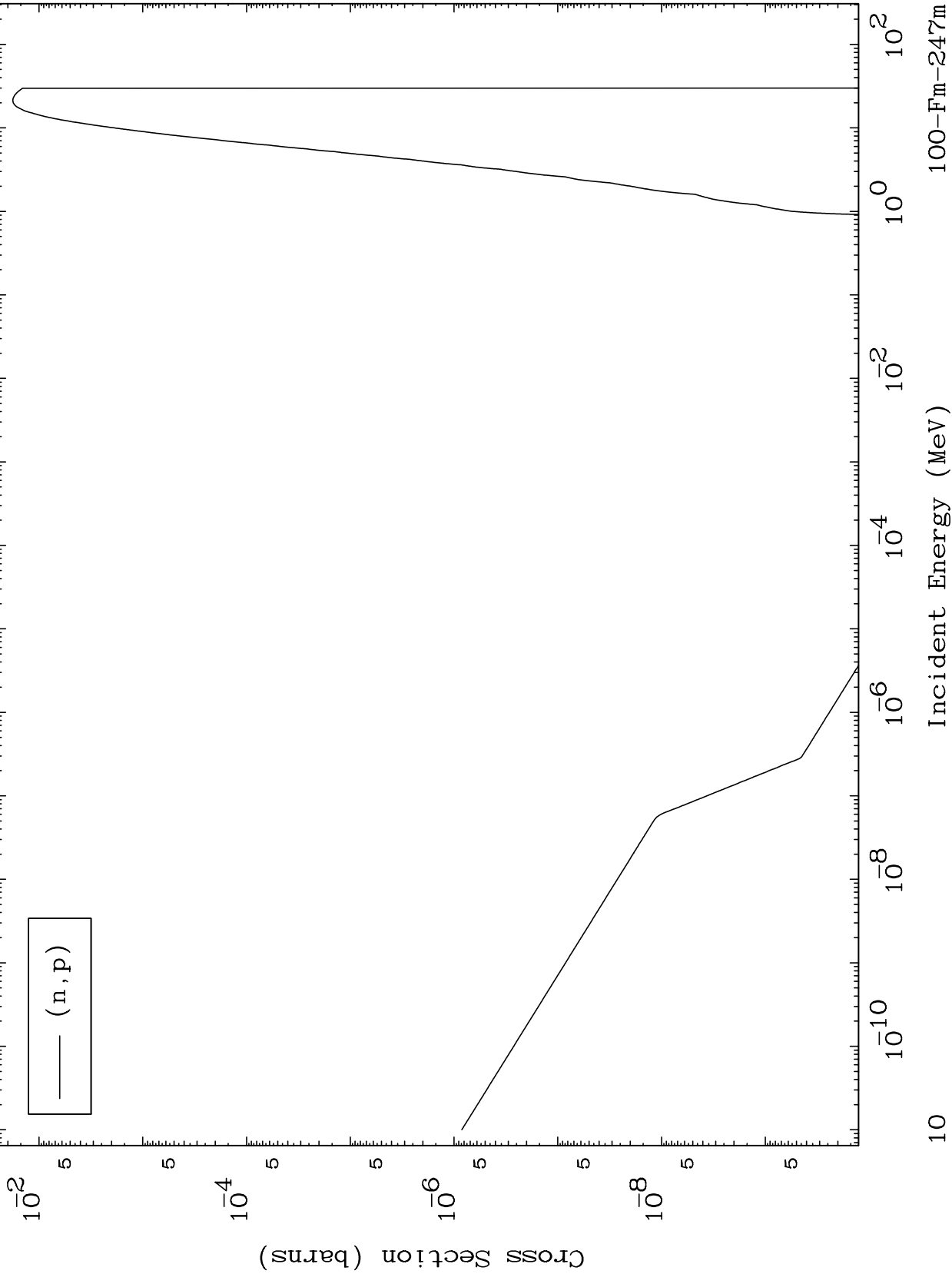
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(n,p) Levels
293 Kelvin Cross Sections

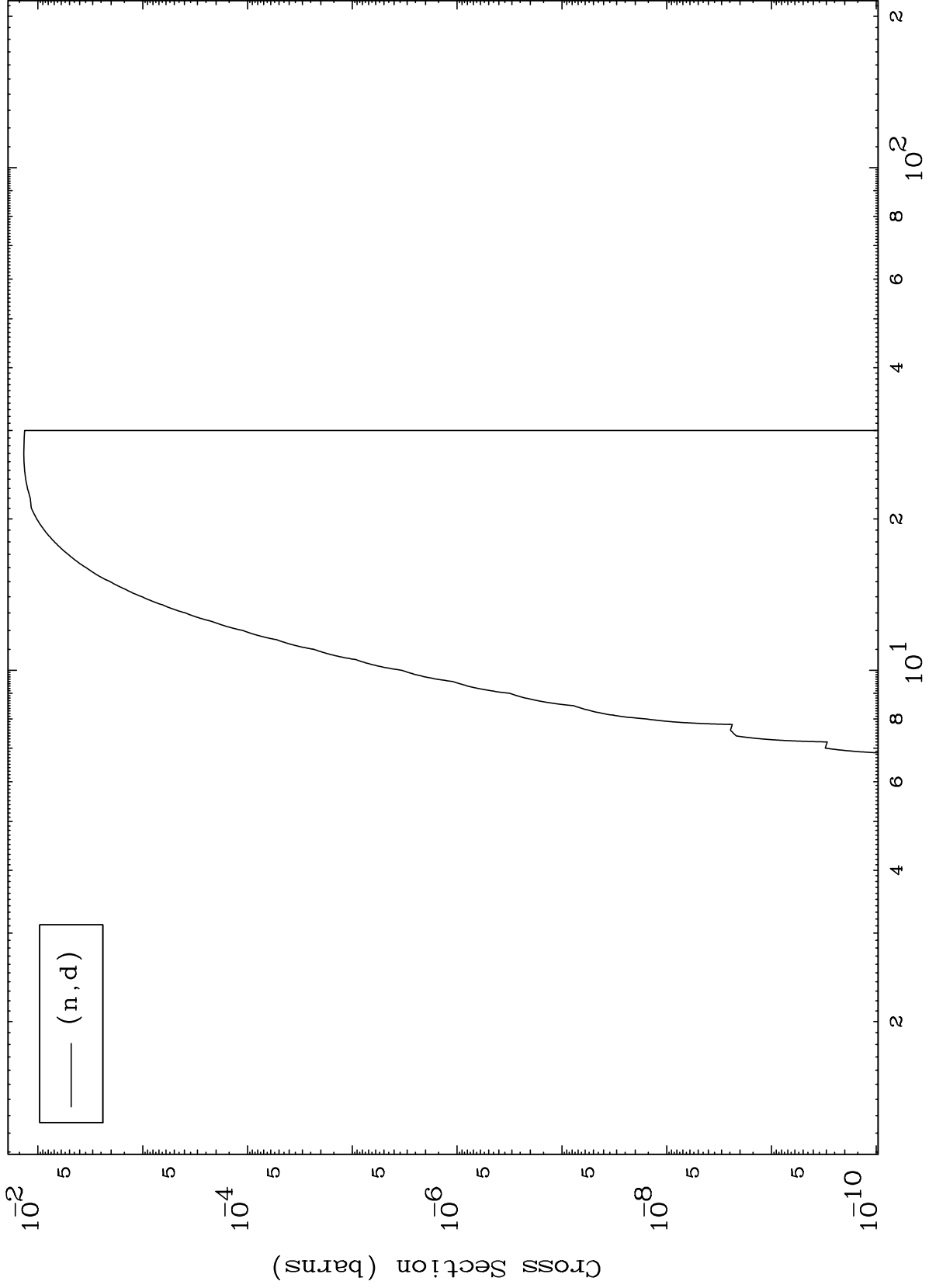
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(n,d) Levels
293 Kelvin Cross Sections

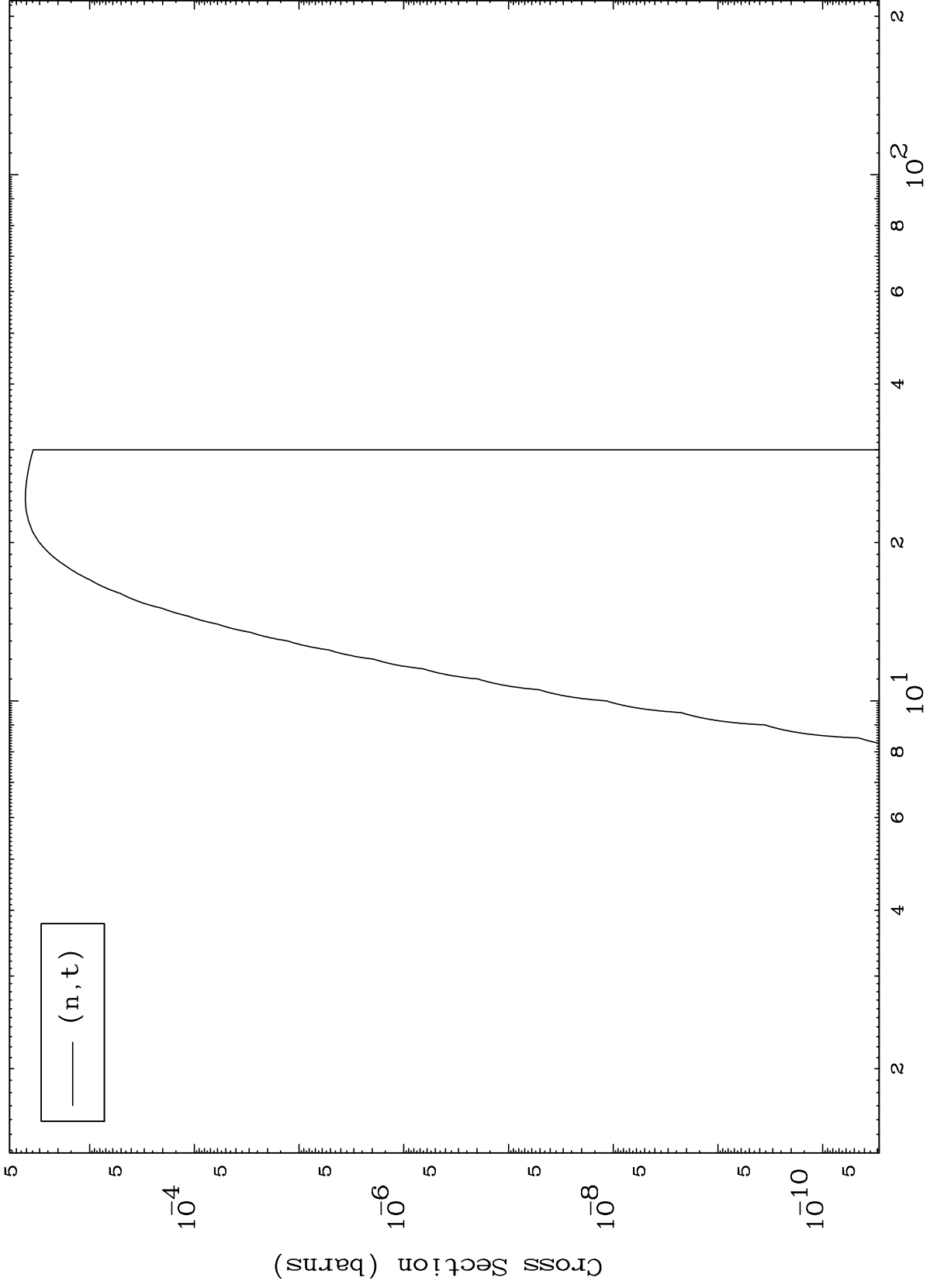
100-Fm-247m



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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-247m



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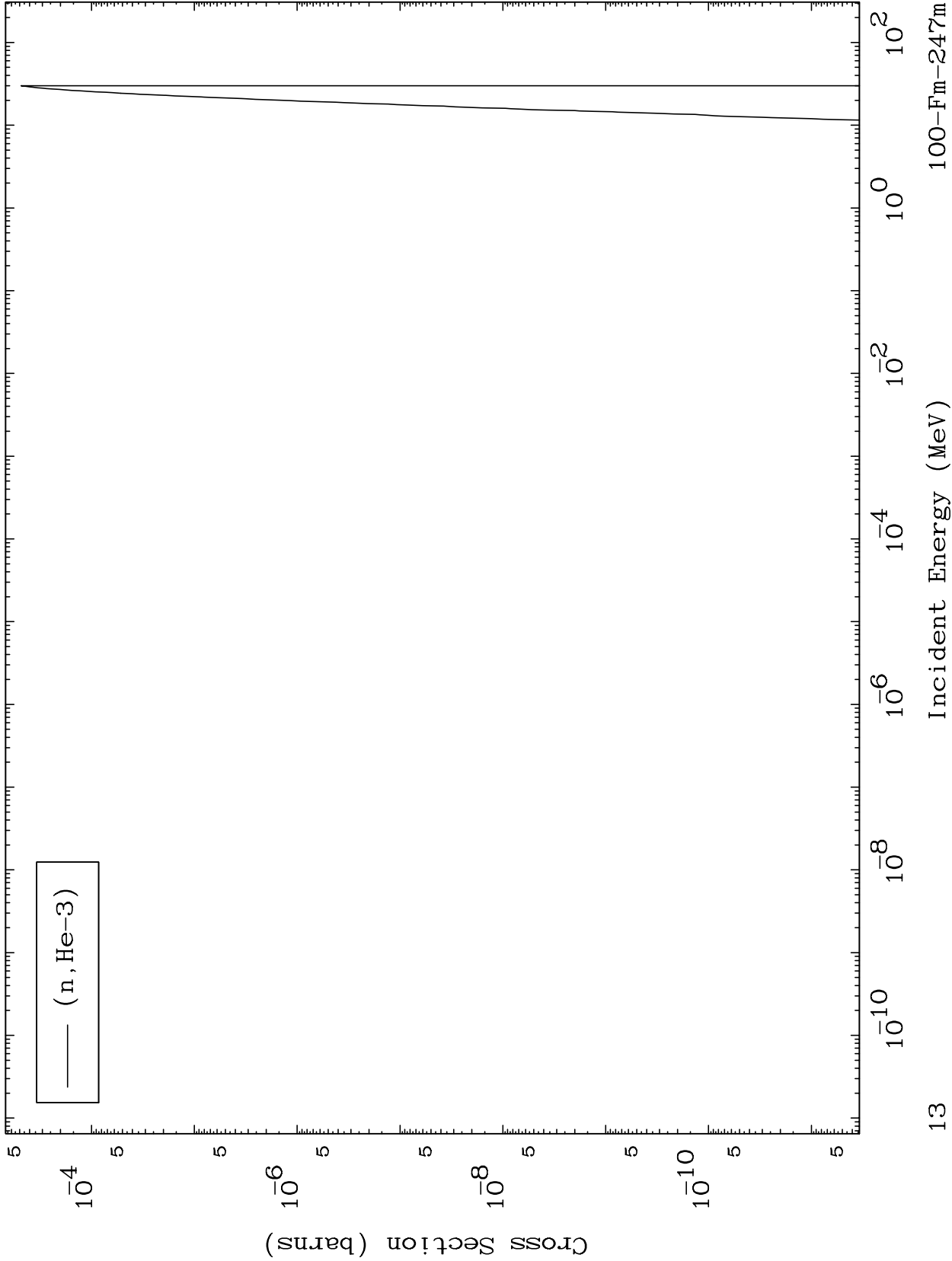
Incident Energy (MeV)

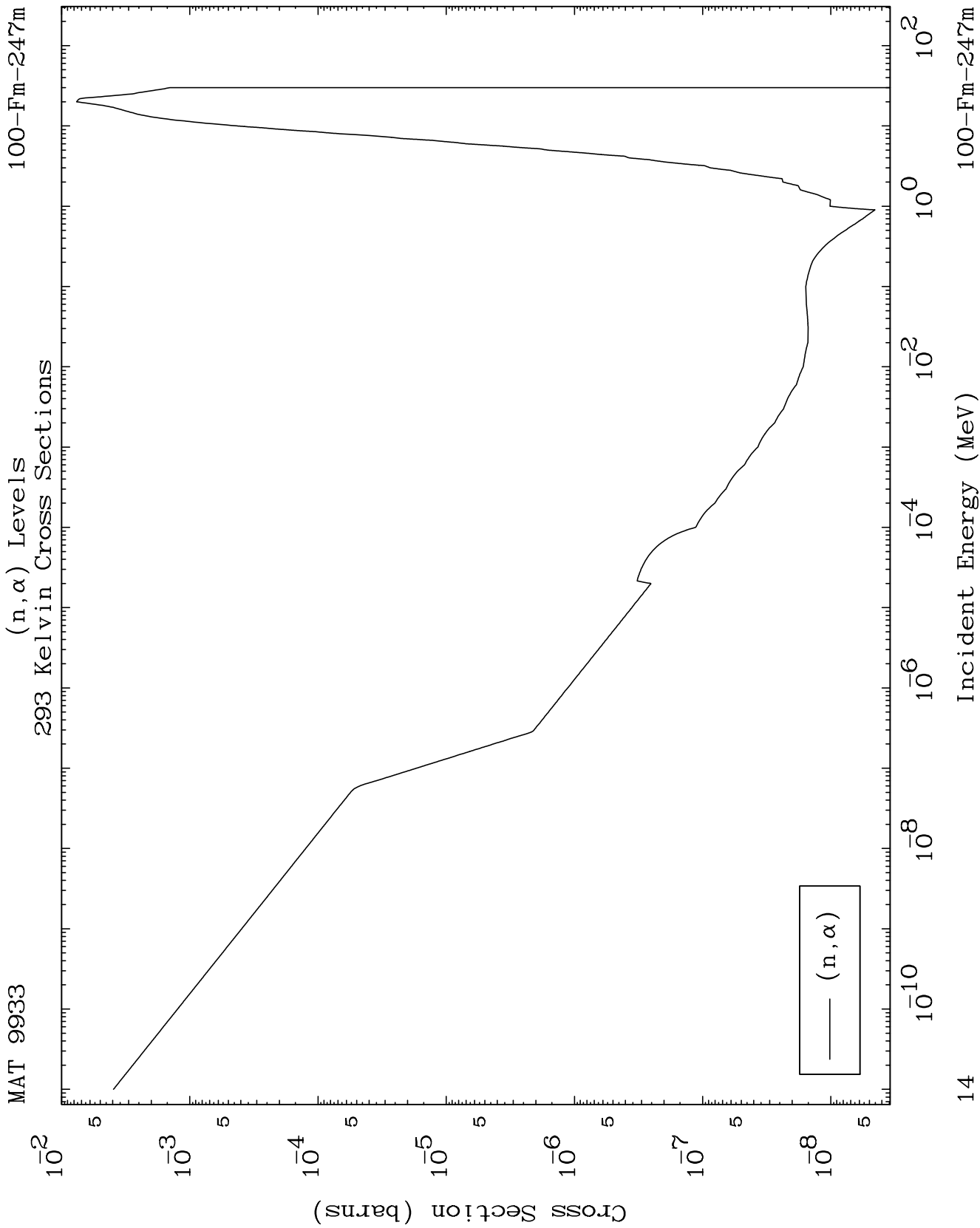
100-Fm-247m

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(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-247m

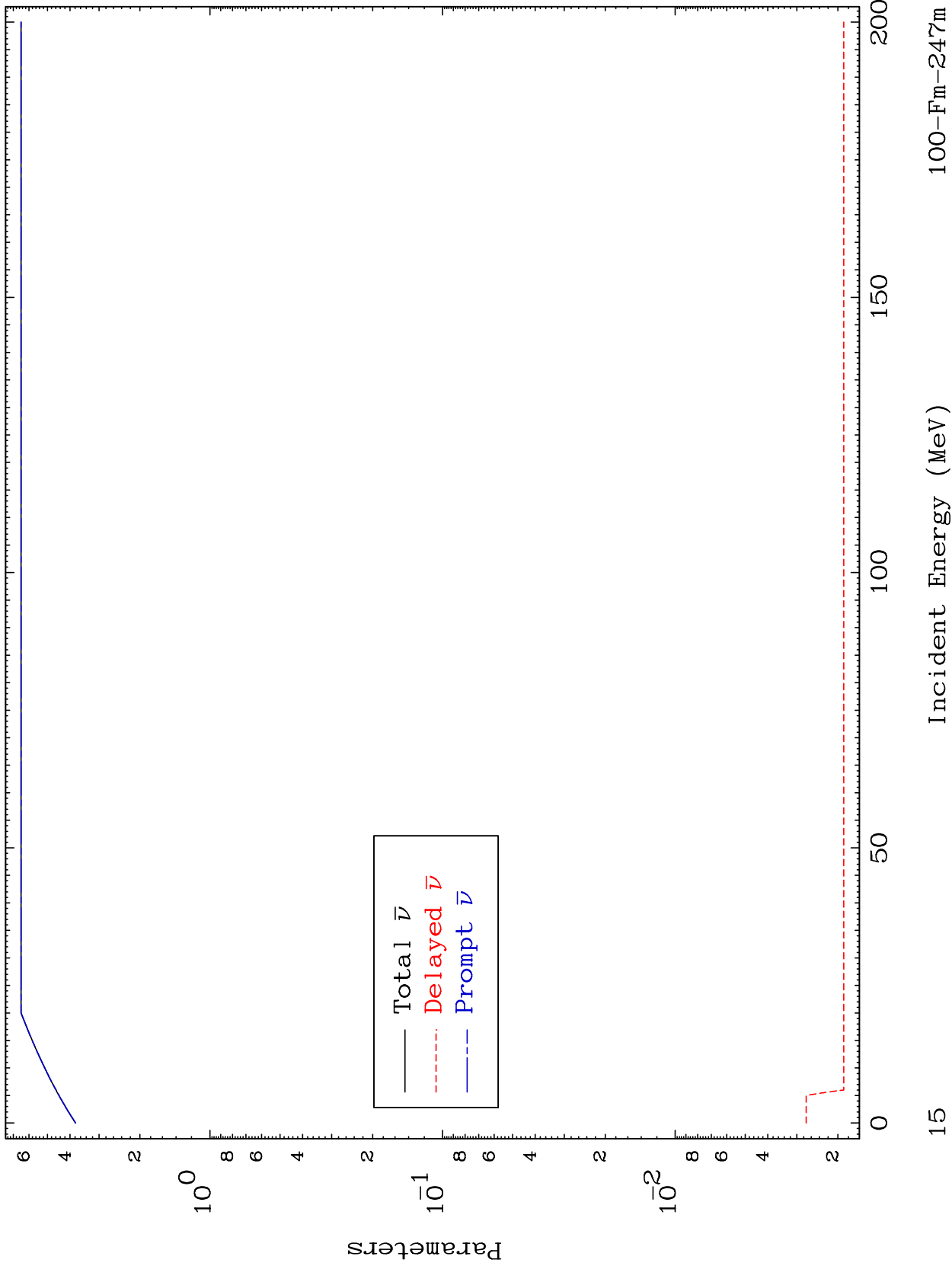




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Energy Release
Parameters

100-Fm-247m



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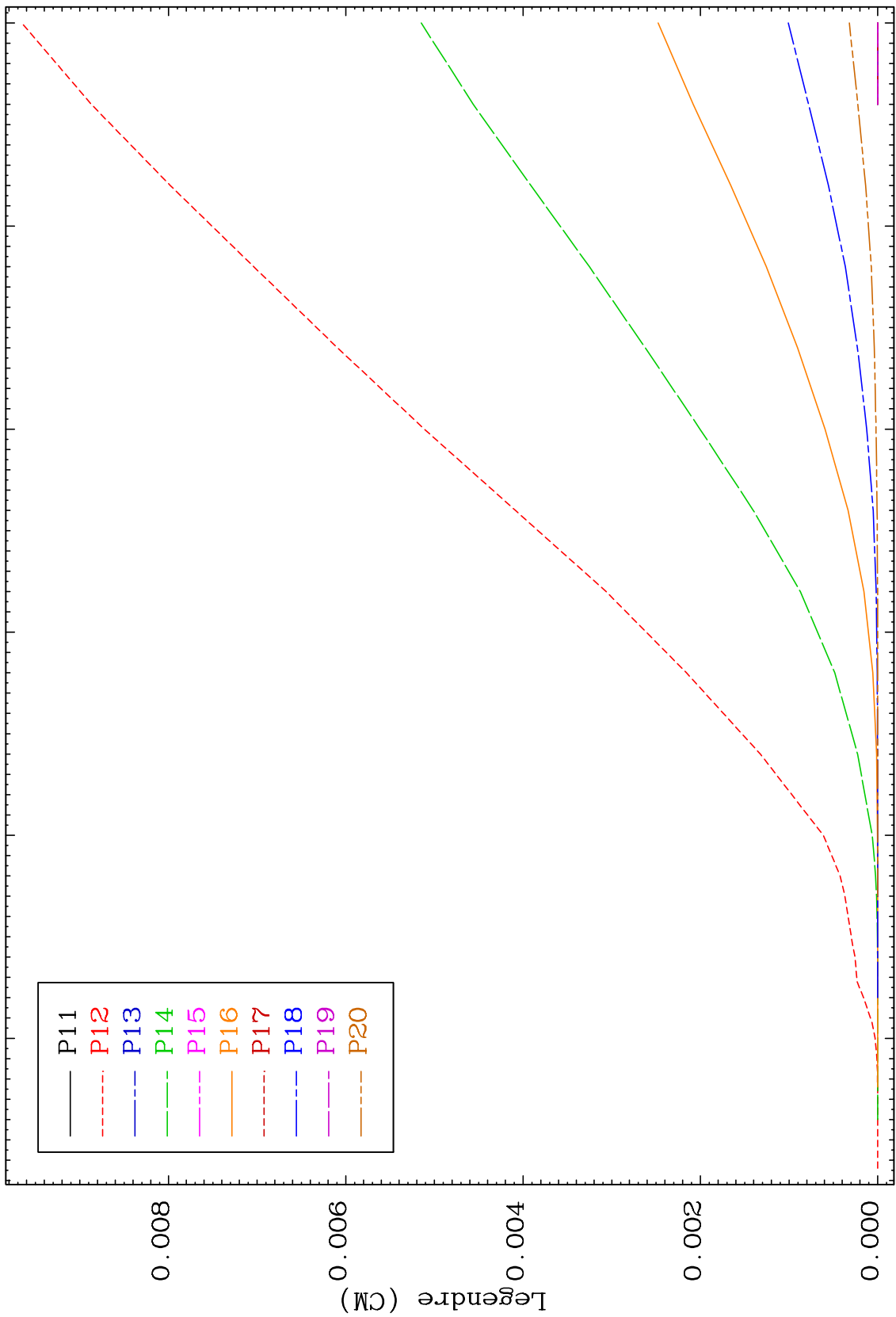
Incident Energy (MeV)

100-Fm-247m

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Elastic Legendre Coefficients

100-Fm-247m



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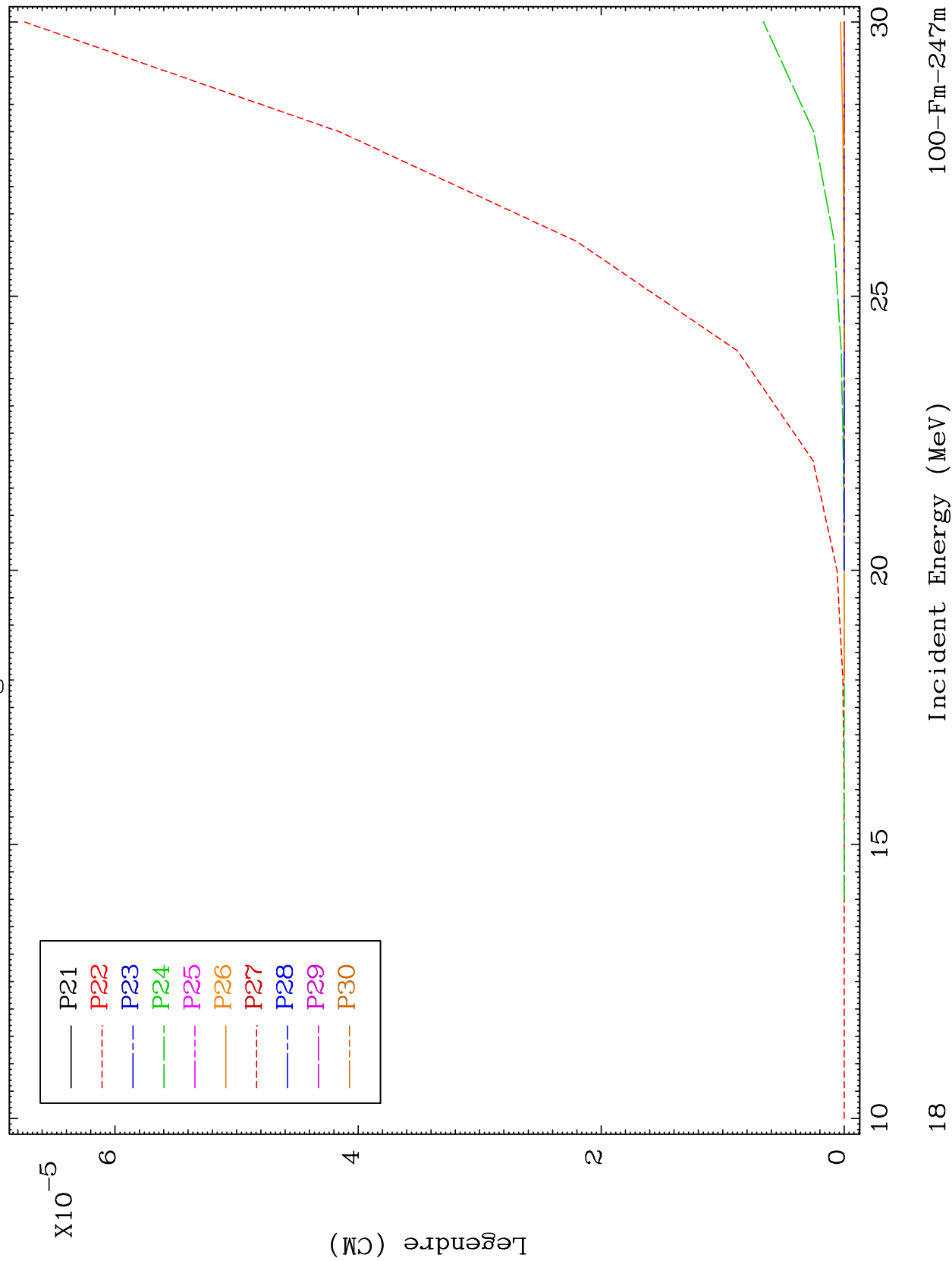
100-Fm-247m

Incident Energy (MeV)

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Elastic
Legendre Coefficients

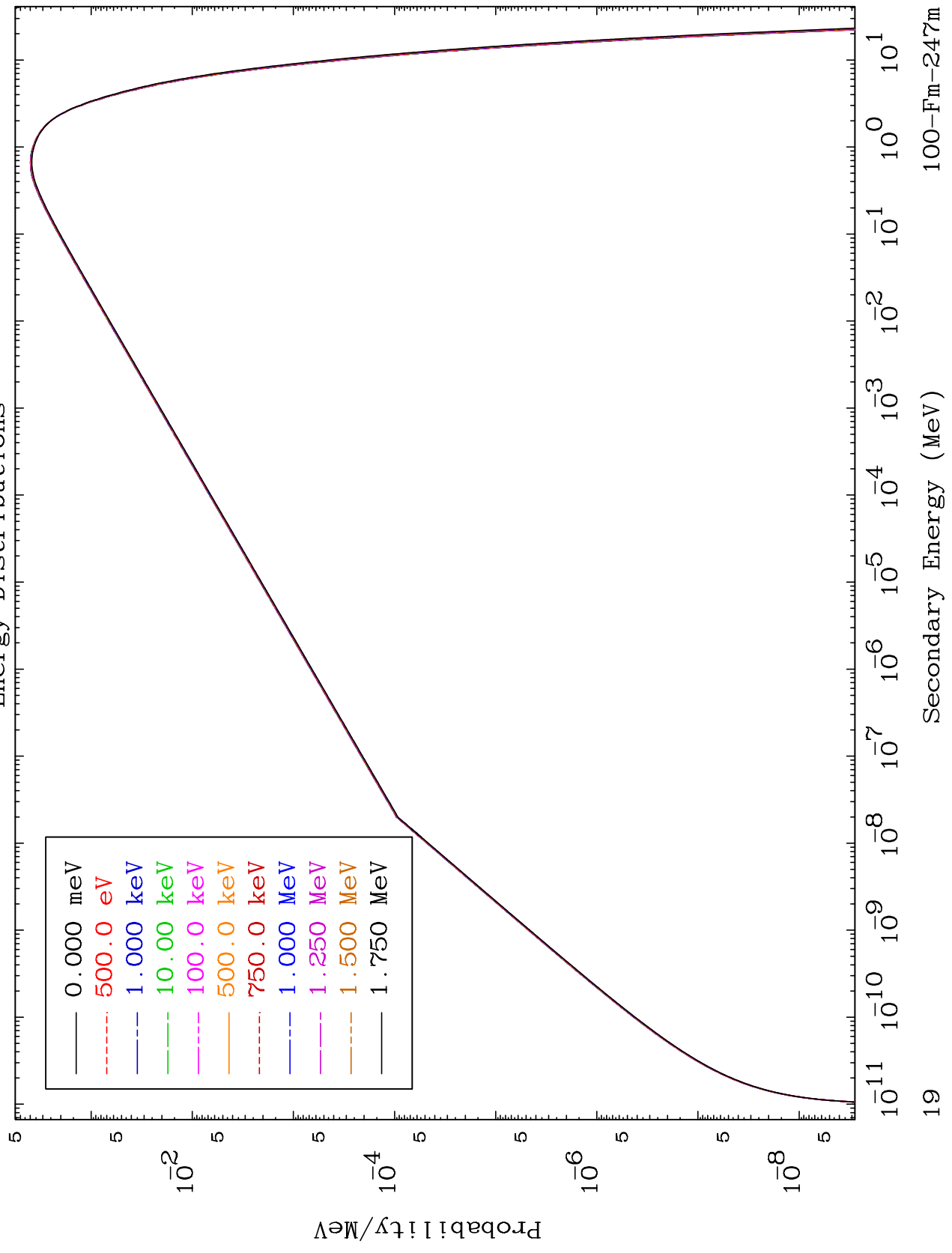
100-Fm-247m



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Fission Energy Distributions

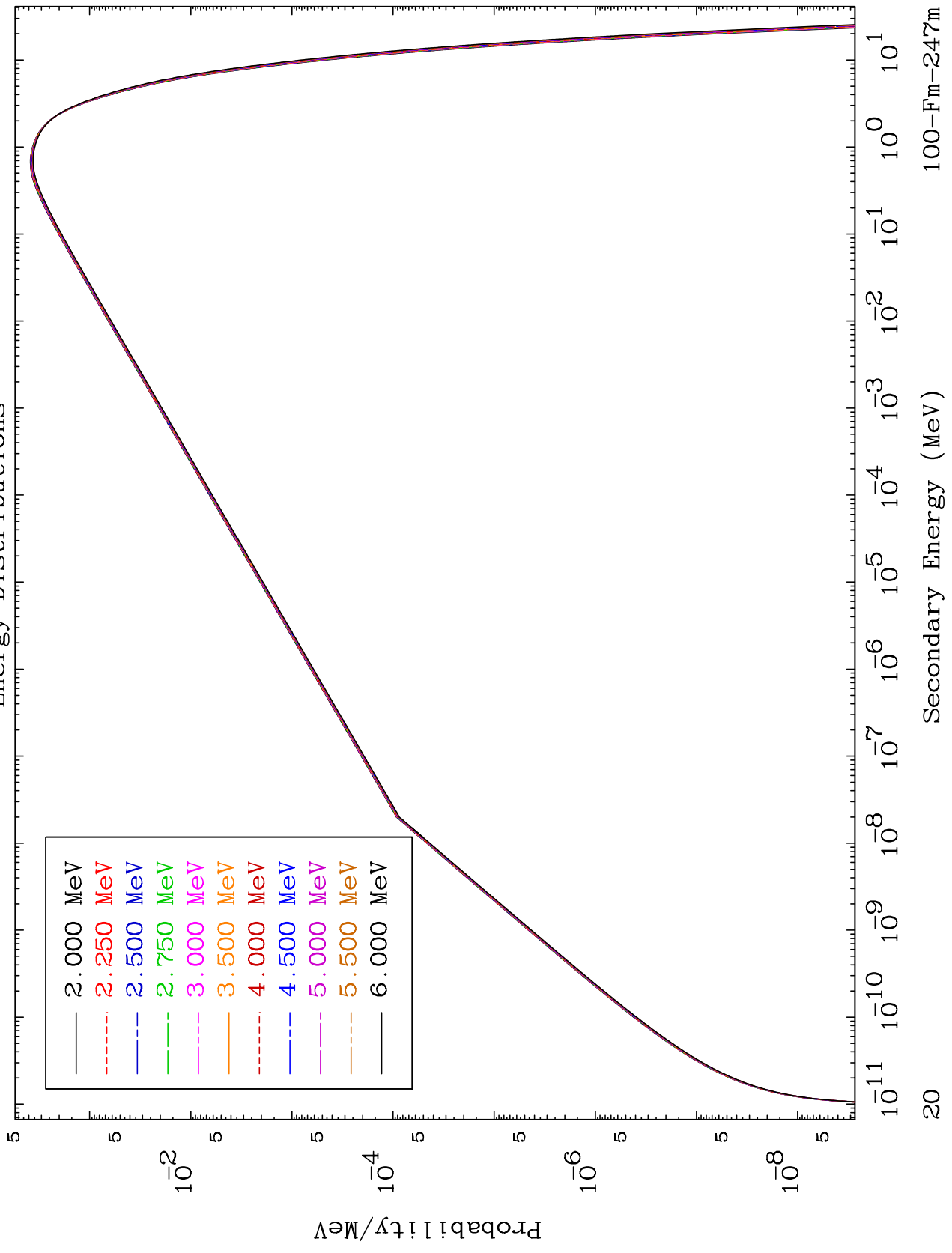
100-Fm-247m



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Fission Energy Distributions

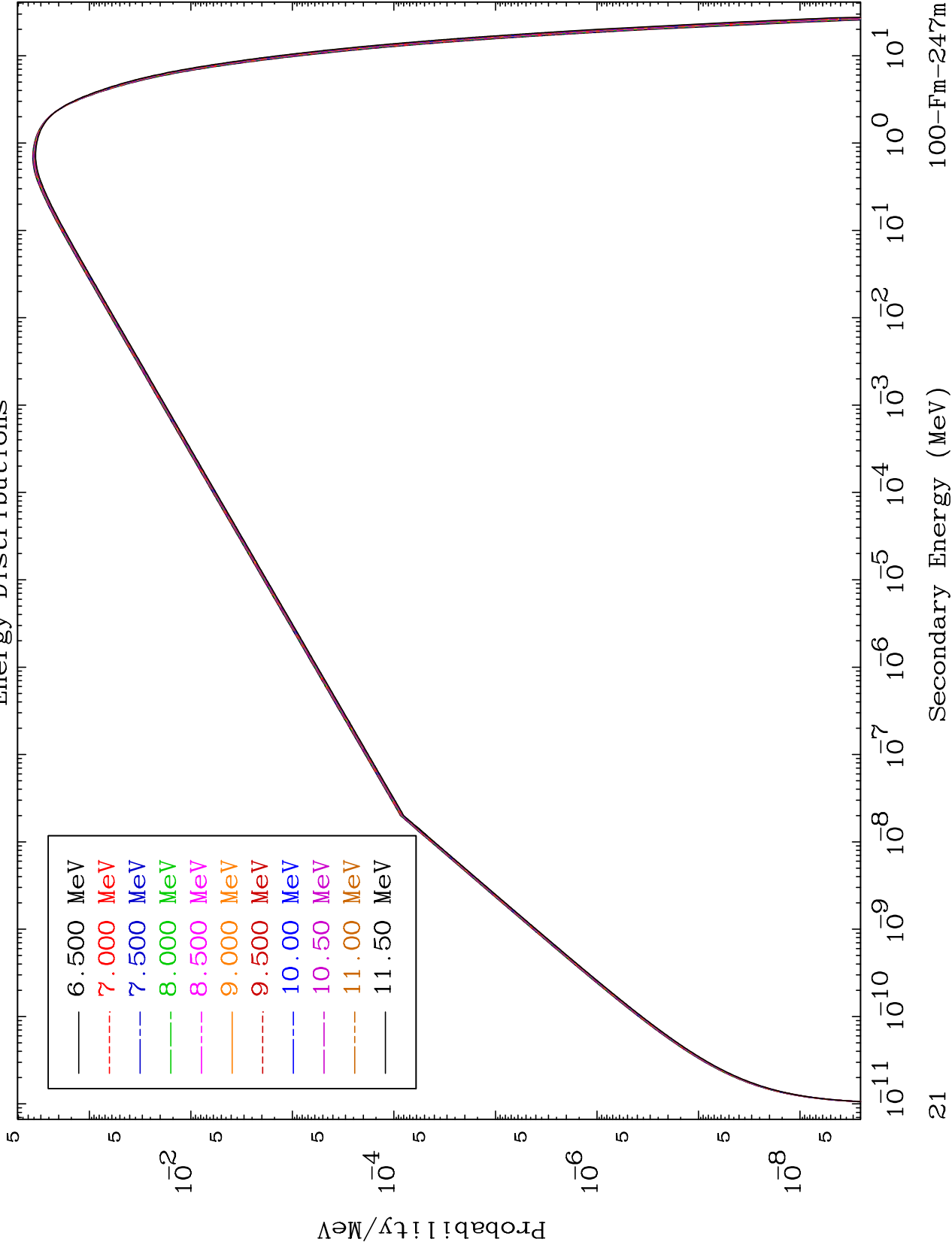
100-Fm-247m



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Fission Energy Distributions

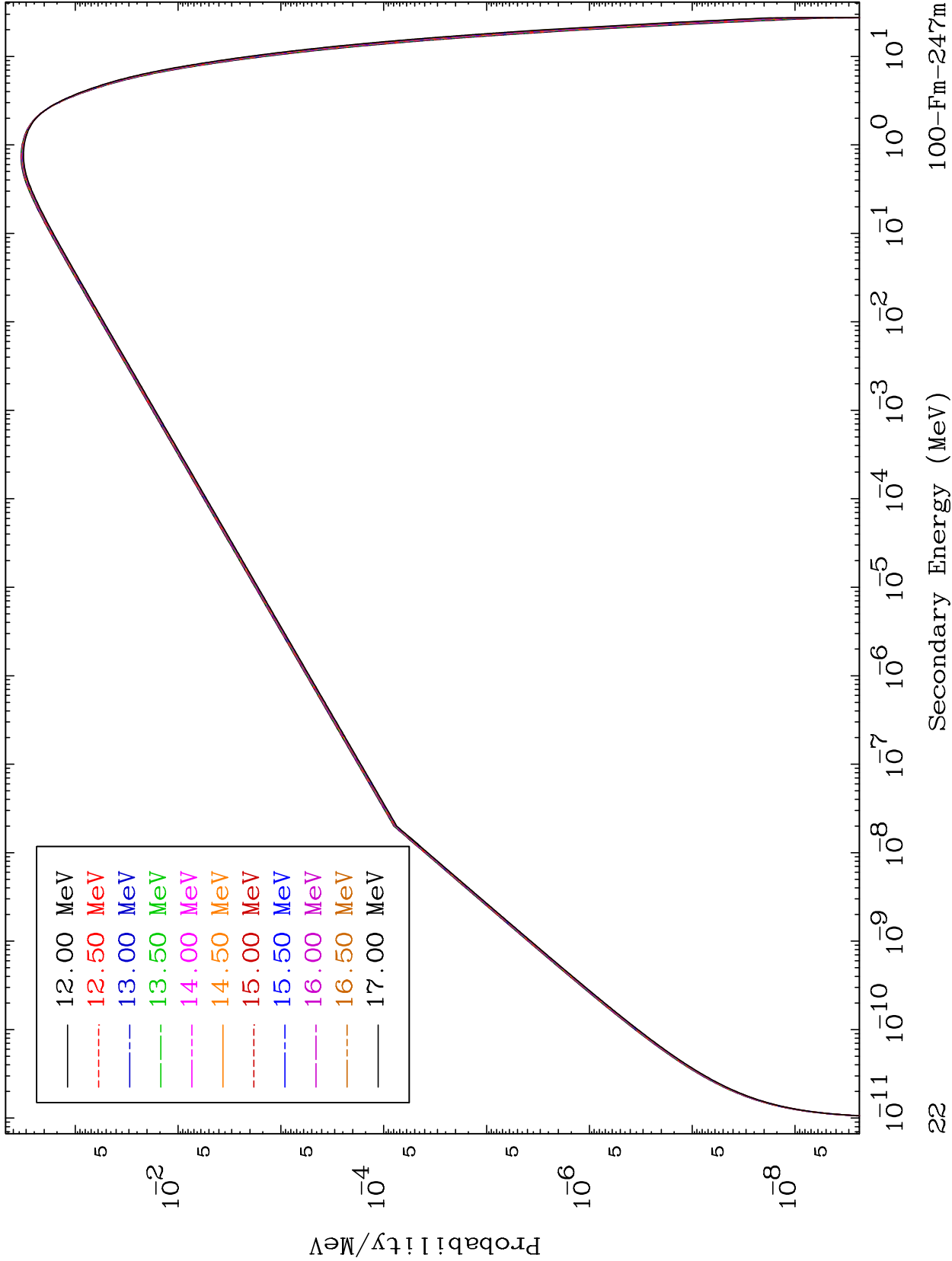
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Fission Energy Distributions

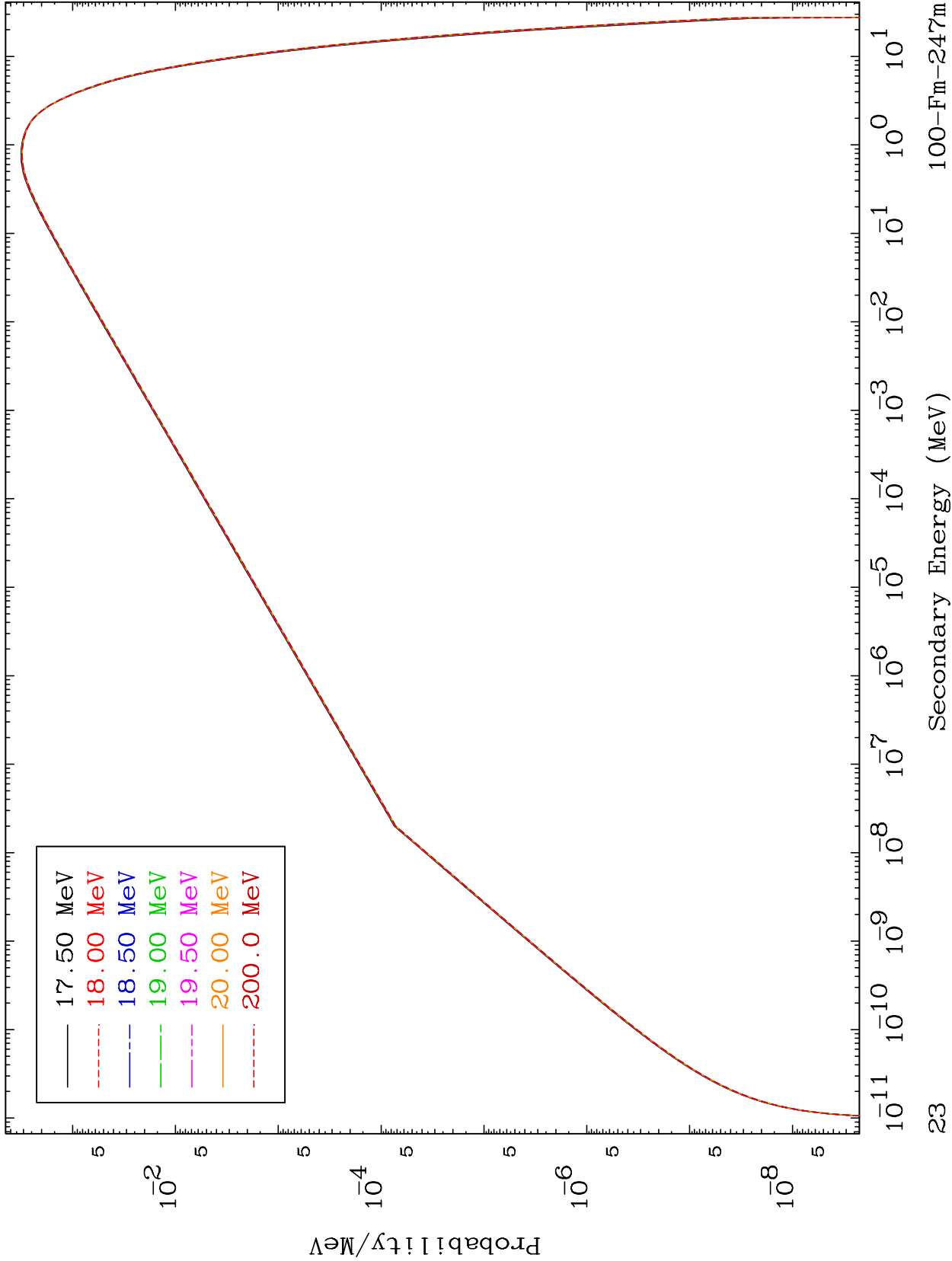
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Fission Energy Distributions

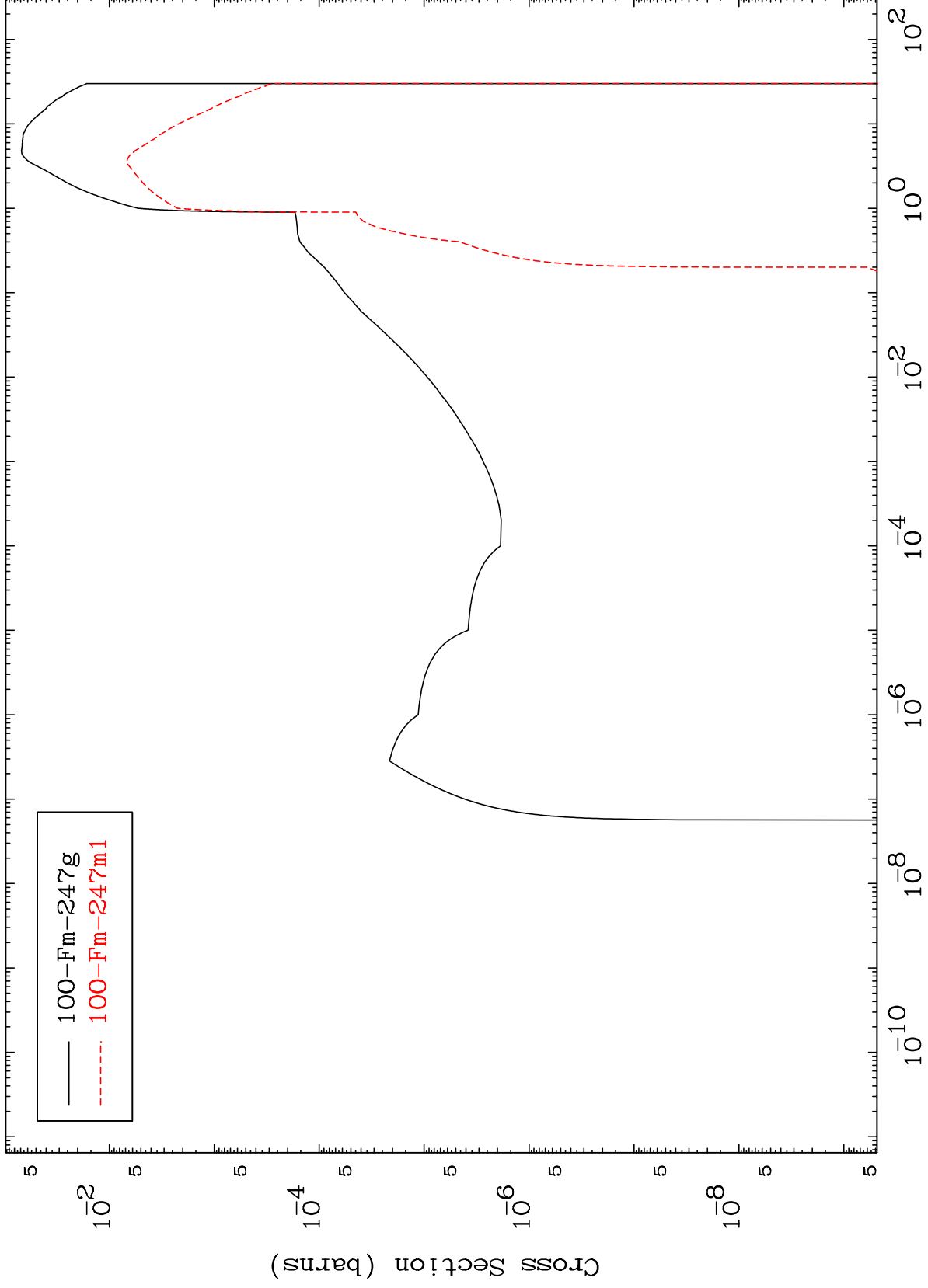
100-Fm-247m



MAT 9933

Inelastic
Radionuclide Production Cross Section

100-Fm-247m



— 100-Fm-247g
- - - 100-Fm-247m1

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(n,p)

100-Fm-247m

Radionuclide Production Cross Section

