

Program EVALPLOT
(Version 2021-1)

by

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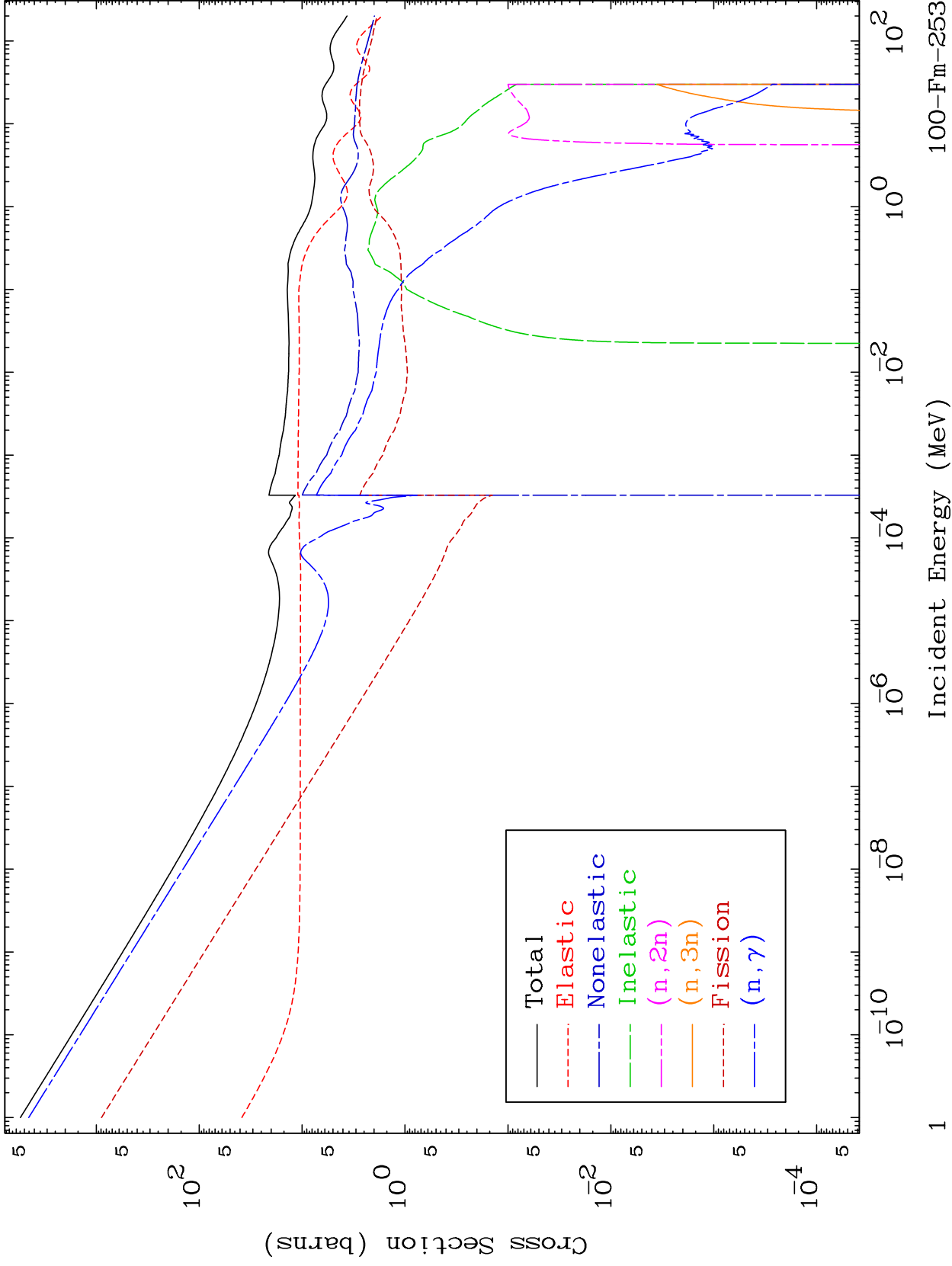
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9934

Neutron Major
293 Kelvin Cross Sections

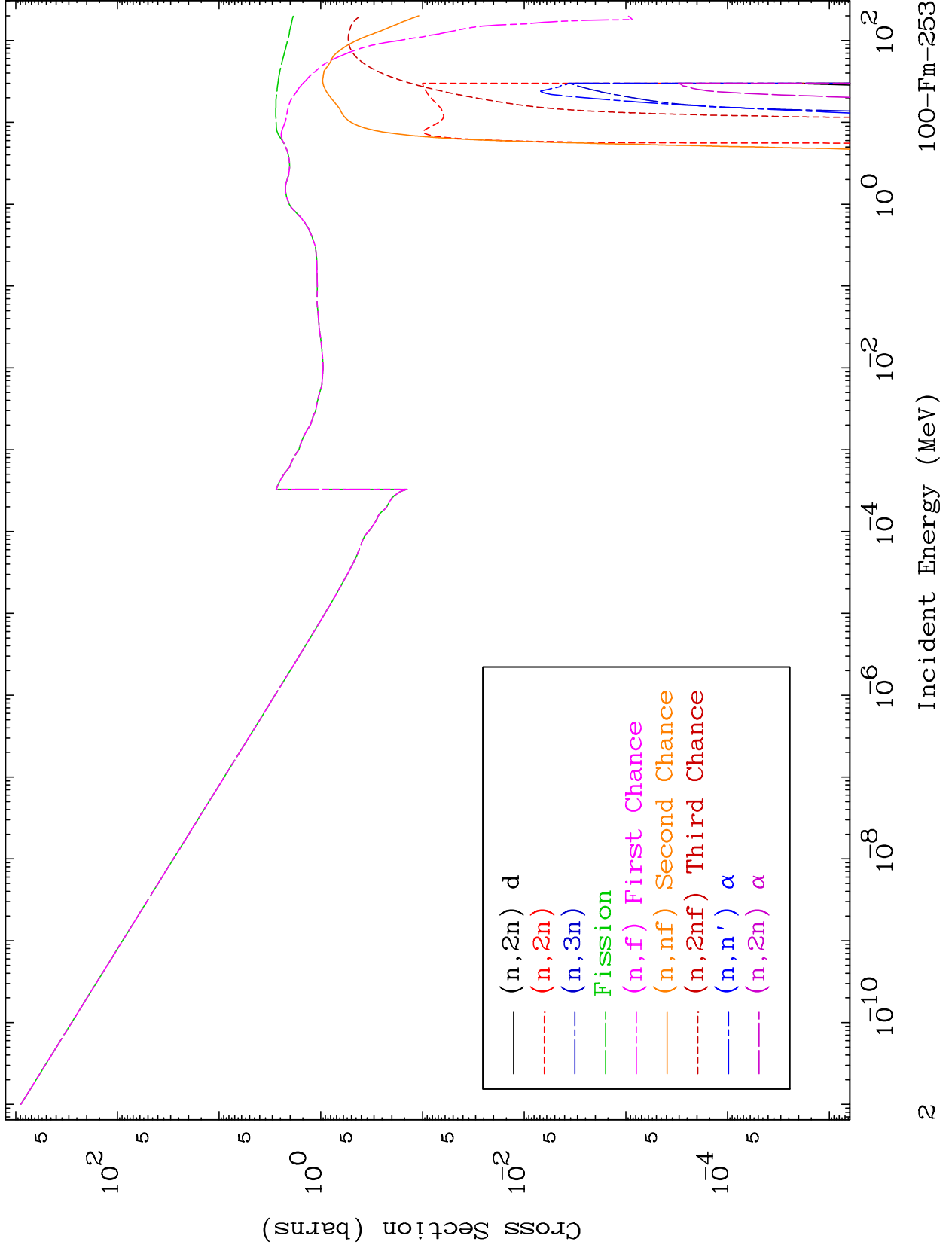
100-Fm-253



MAT 9934

Neutron Absorption
293 Kelvin Cross Sections

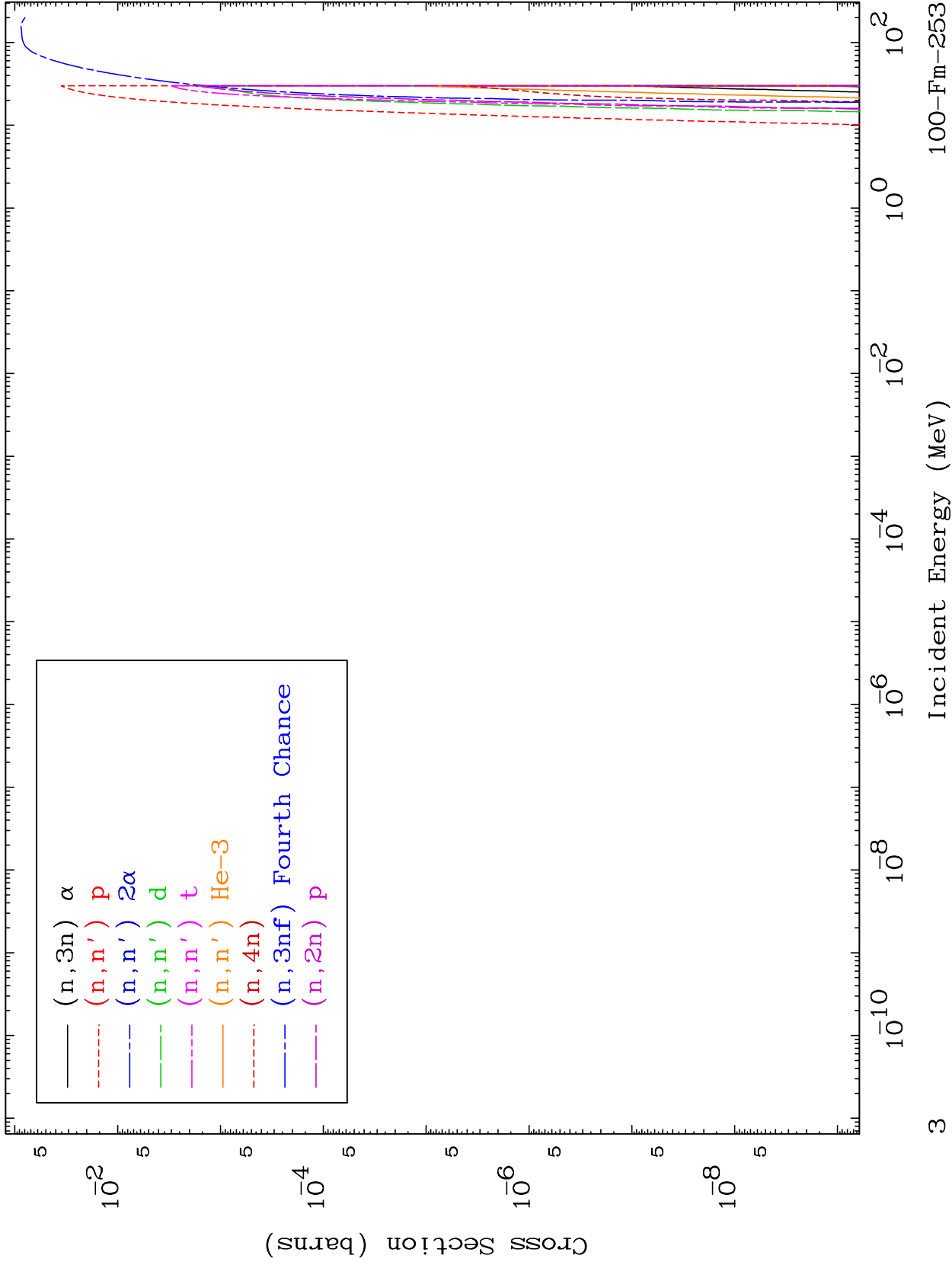
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MAT 9934

Neutron Absorption
293 Kelvin Cross Sections

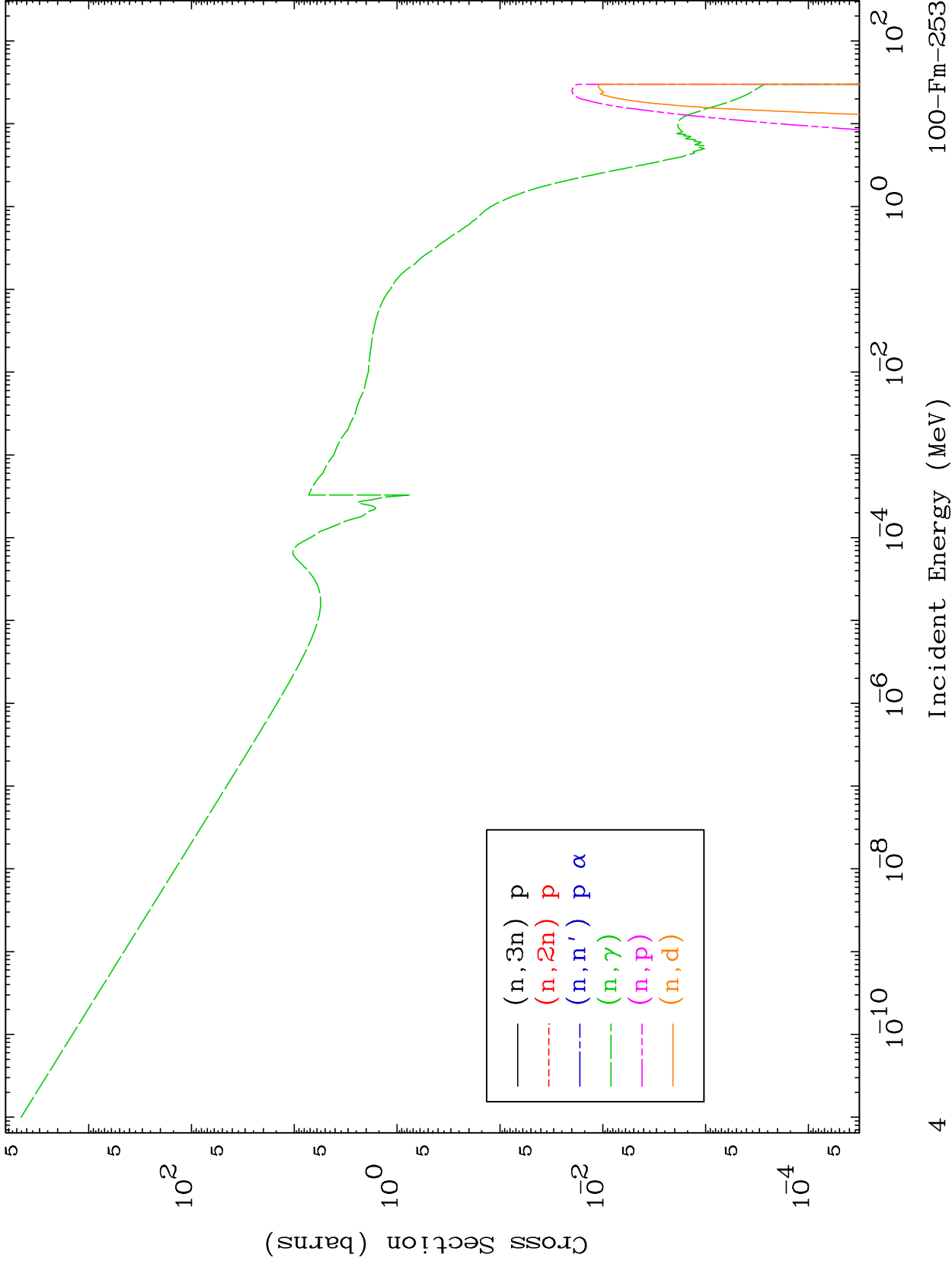
100-Fm-253



MAT 9934

Neutron Absorption
293 Kelvin Cross Sections

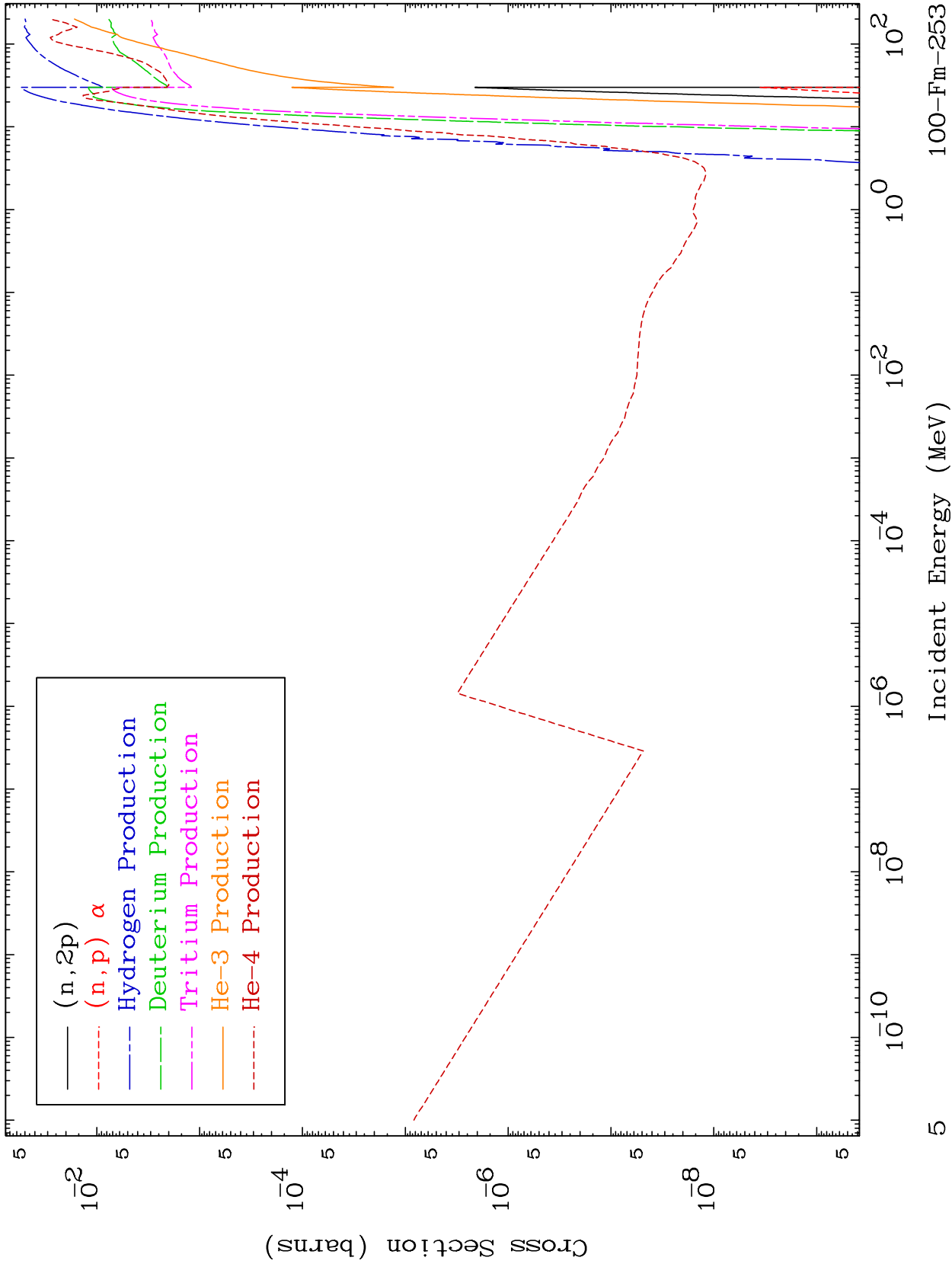
100-Fm-253



MAT 9934

Neutron Absorption
293 Kelvin Cross Sections

100-Fm-253



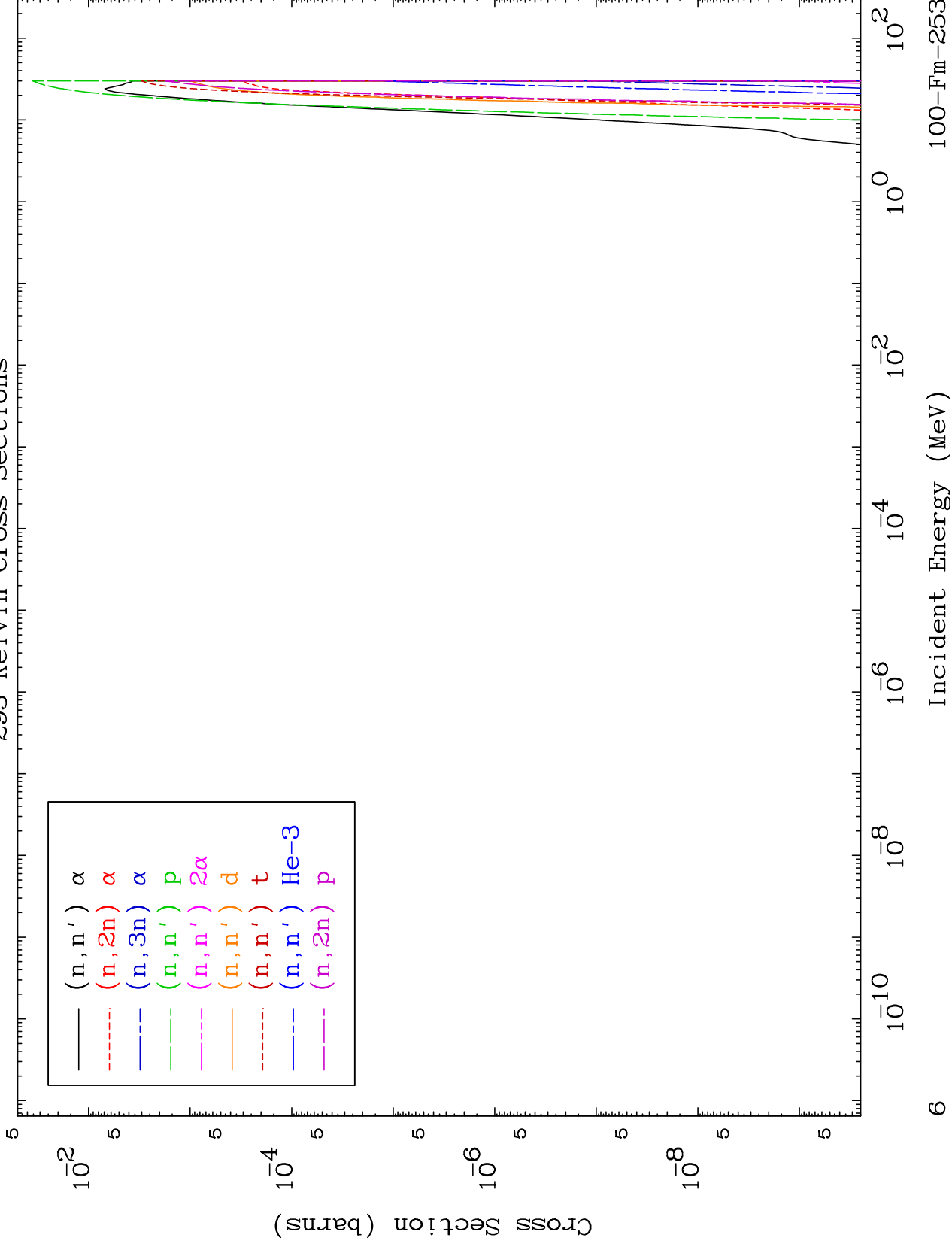
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100-Fm-253

MAT 9934

Charged Particle
293 Kelvin Cross Sections

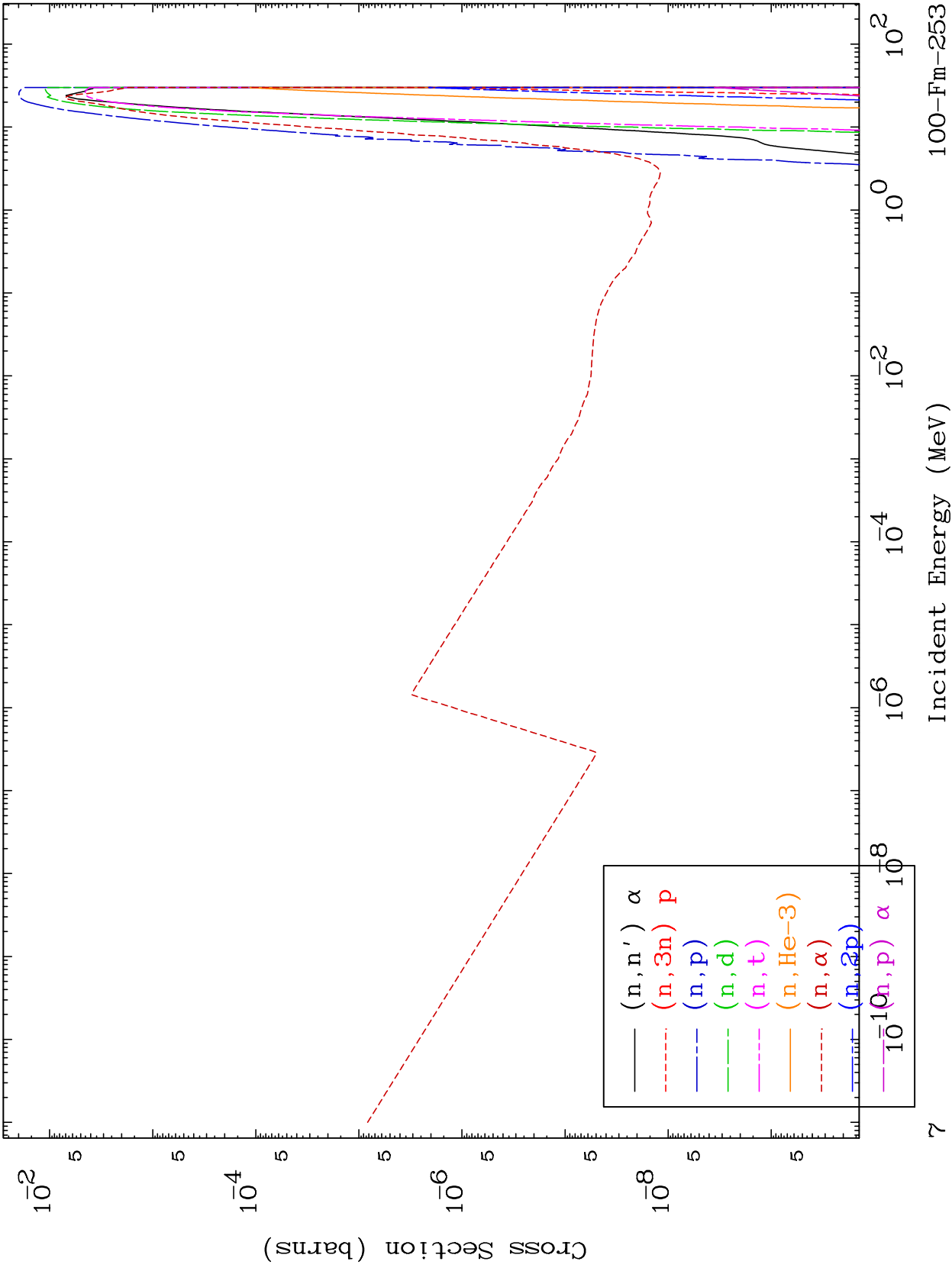
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MAT 9934

Charged Particle
293 Kelvin Cross Sections

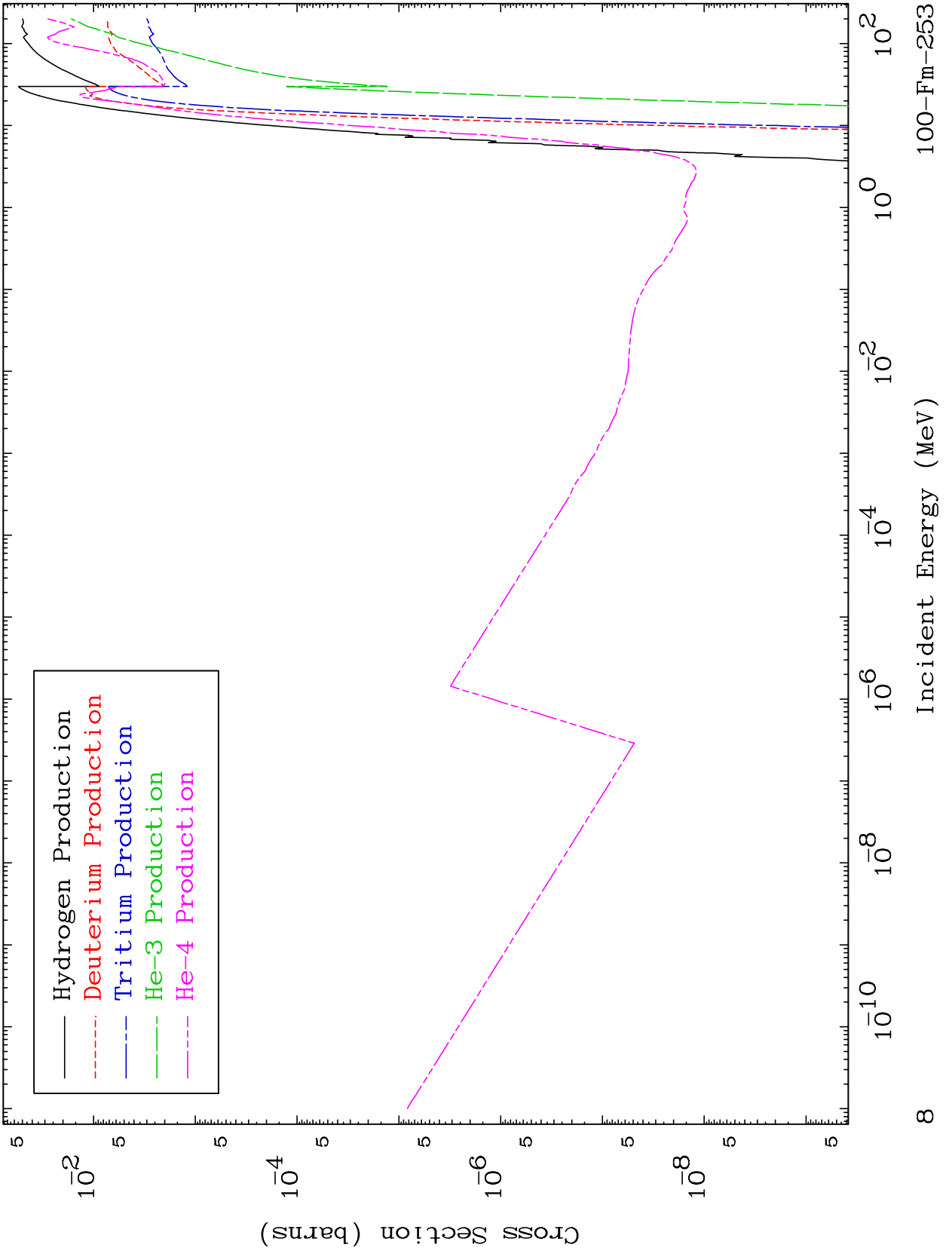
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Particle Production
293 Kelvin Cross Sections

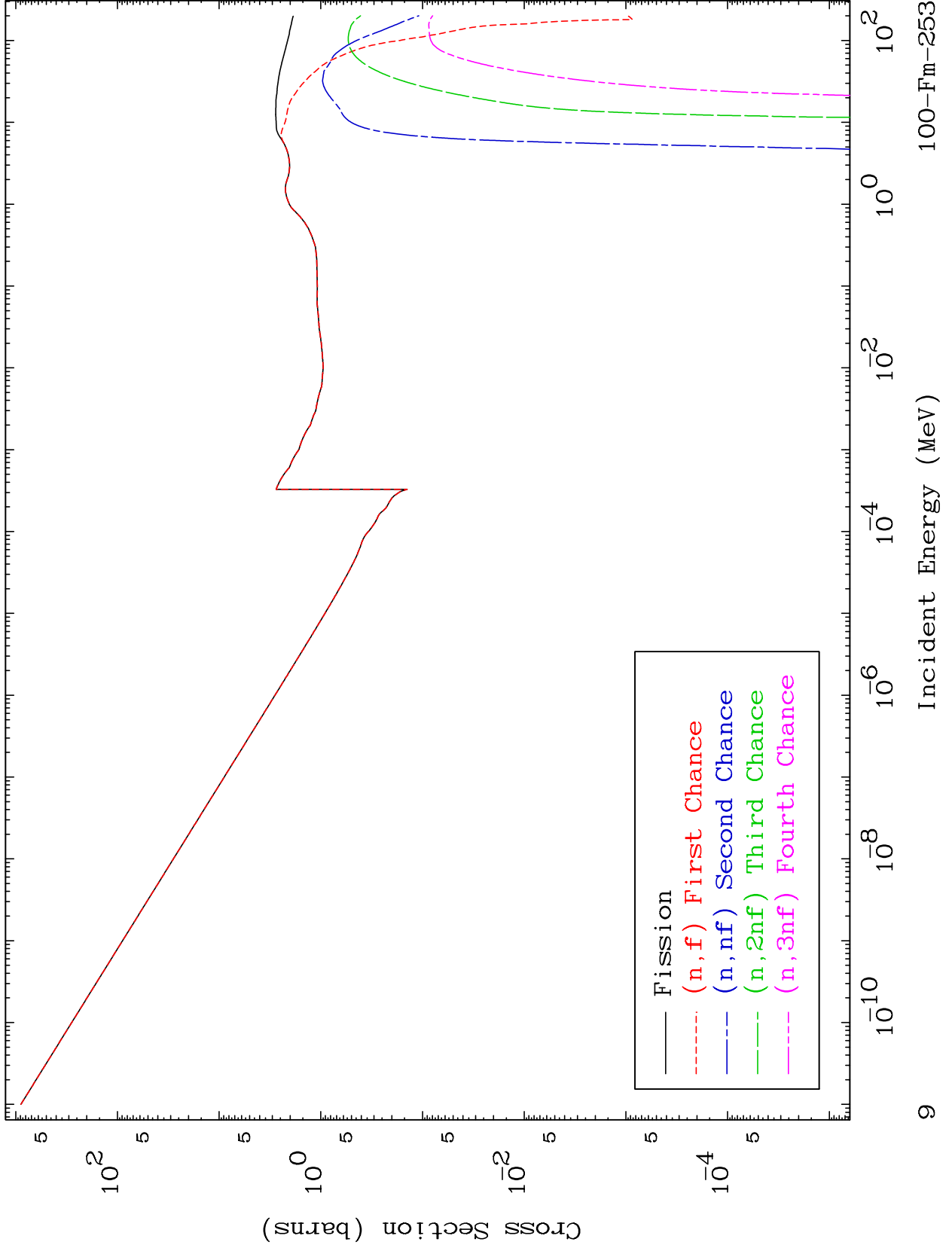
100-Fm-253



MAT 9934

Fission
293 Kelvin Cross Sections

100-Fm-253

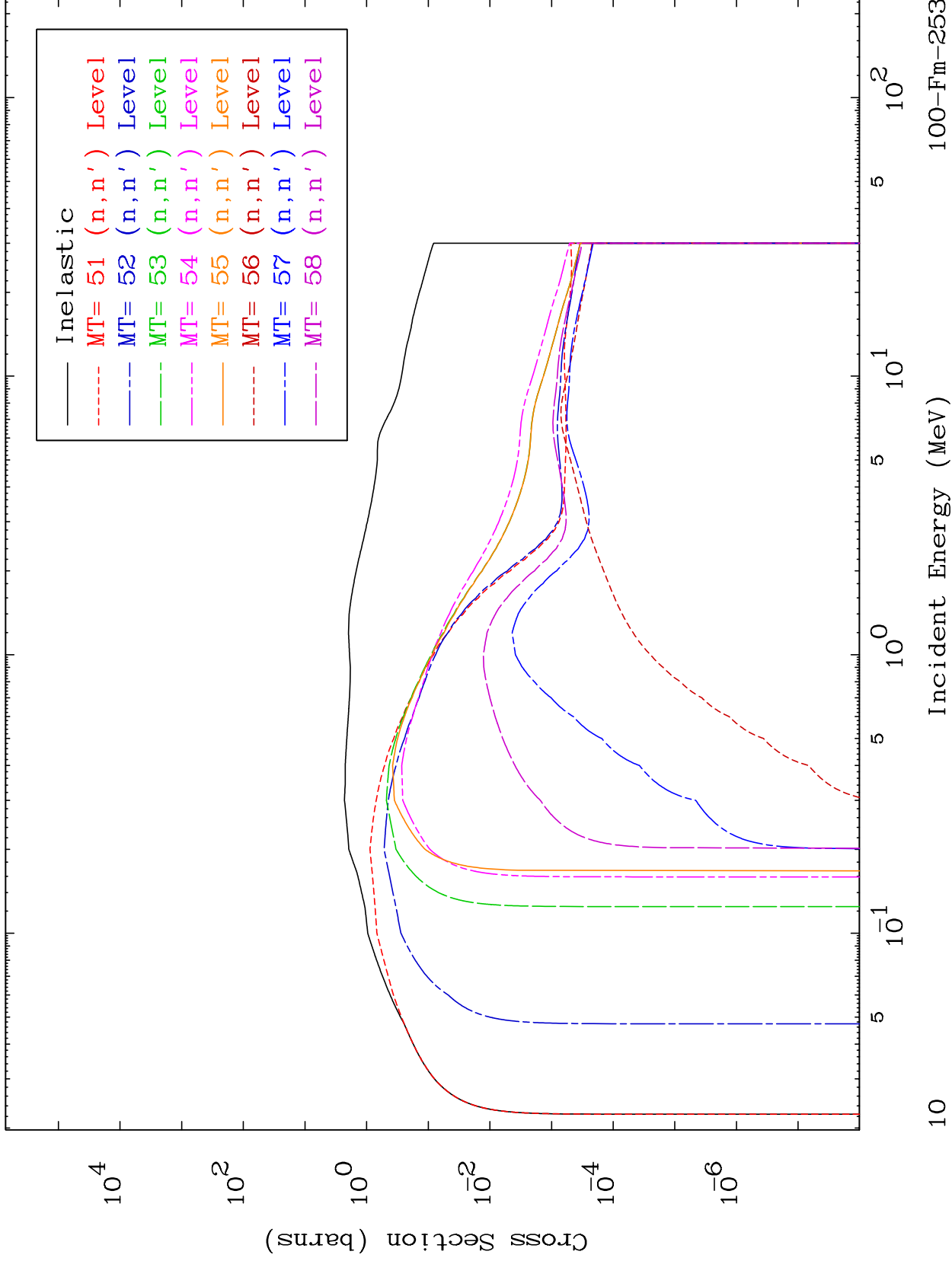


MAT 9934

(n,n') Levels

100-Fm-253

293 Kelvin Cross Sections



10

Incident Energy (MeV)

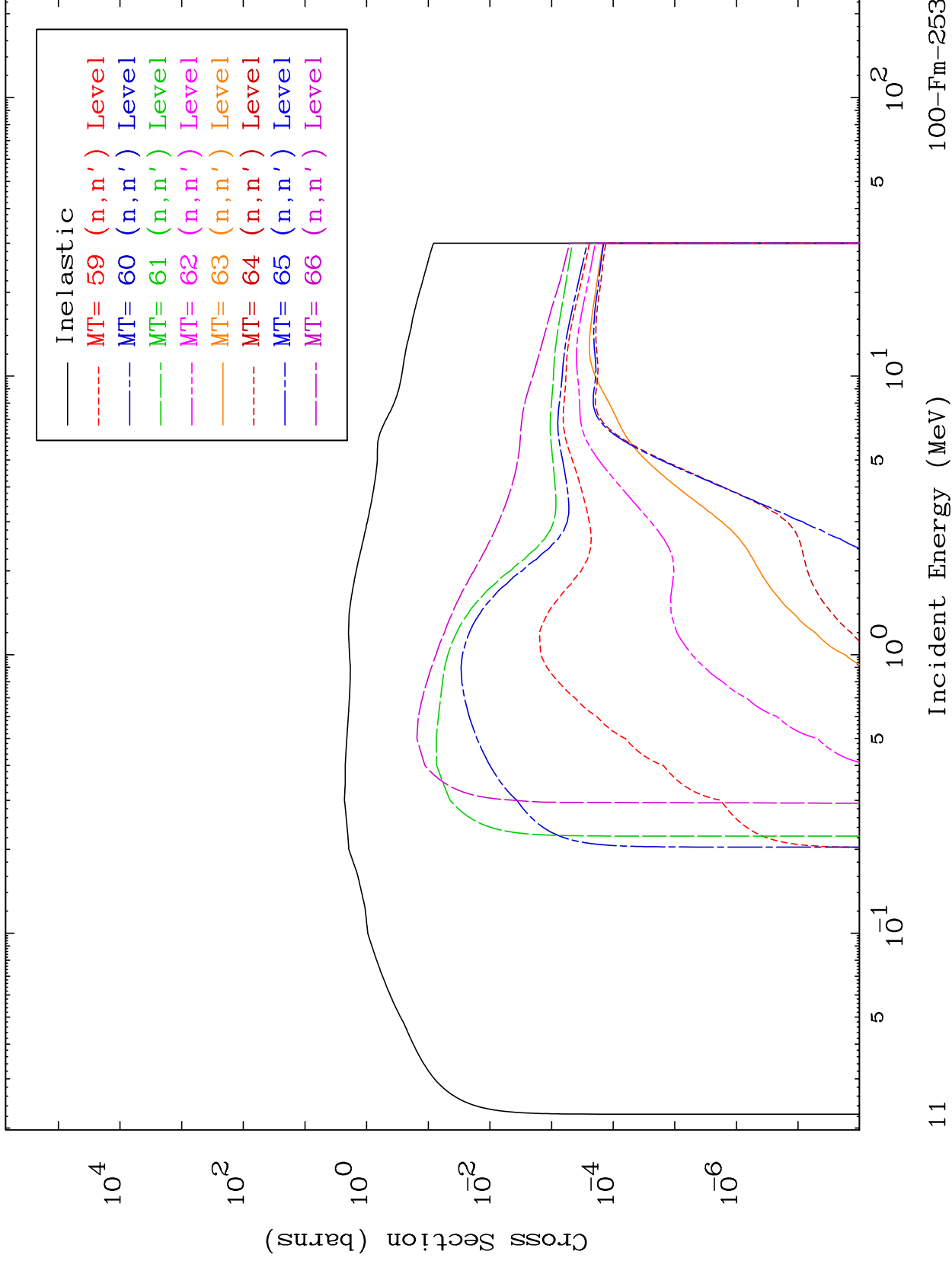
100-Fm-253

MAT 9934

(n,n') Levels

100-Fm-253

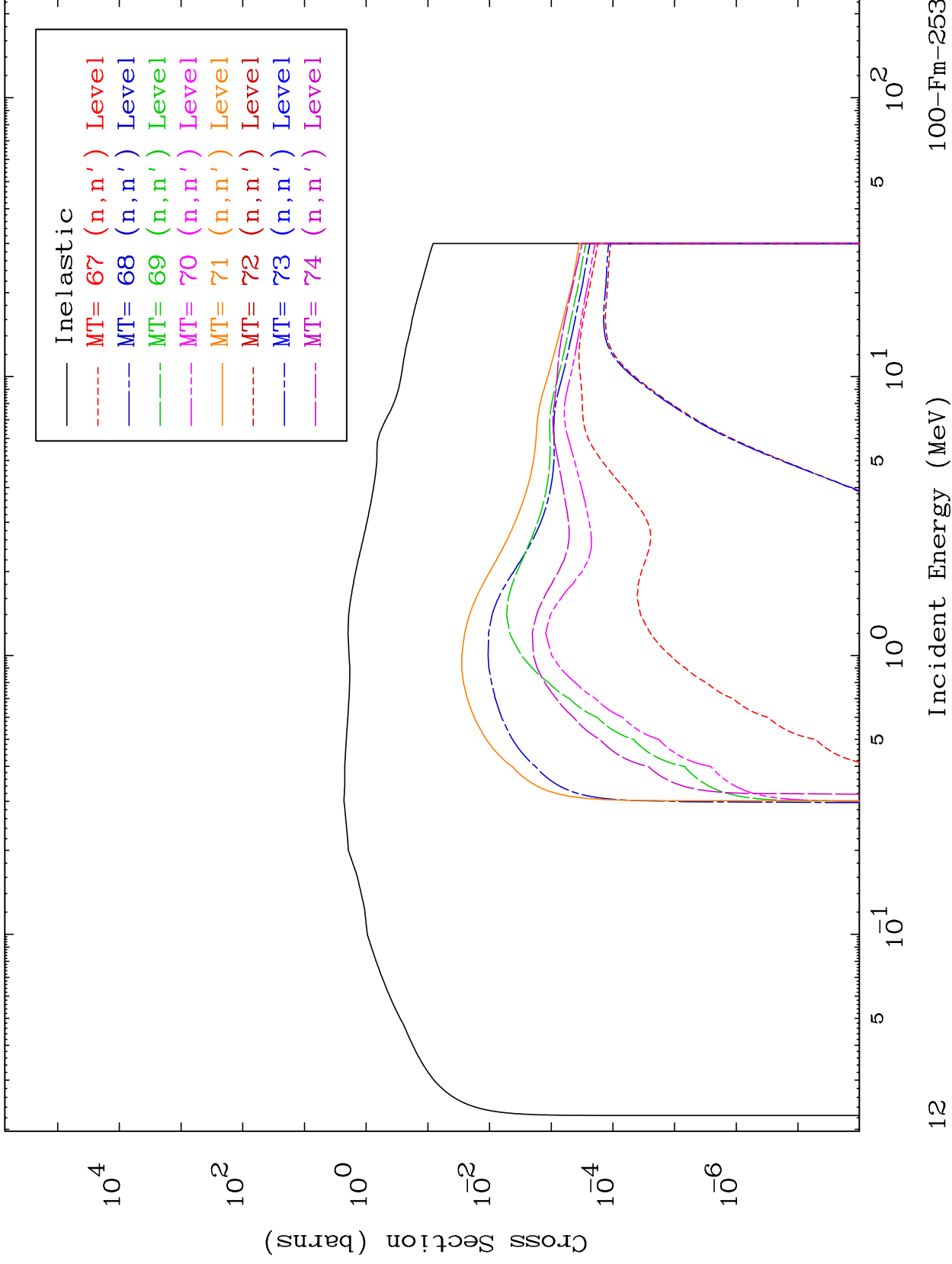
293 Kelvin Cross Sections



MAT 9934

(n,n') Levels
293 Kelvin Cross Sections

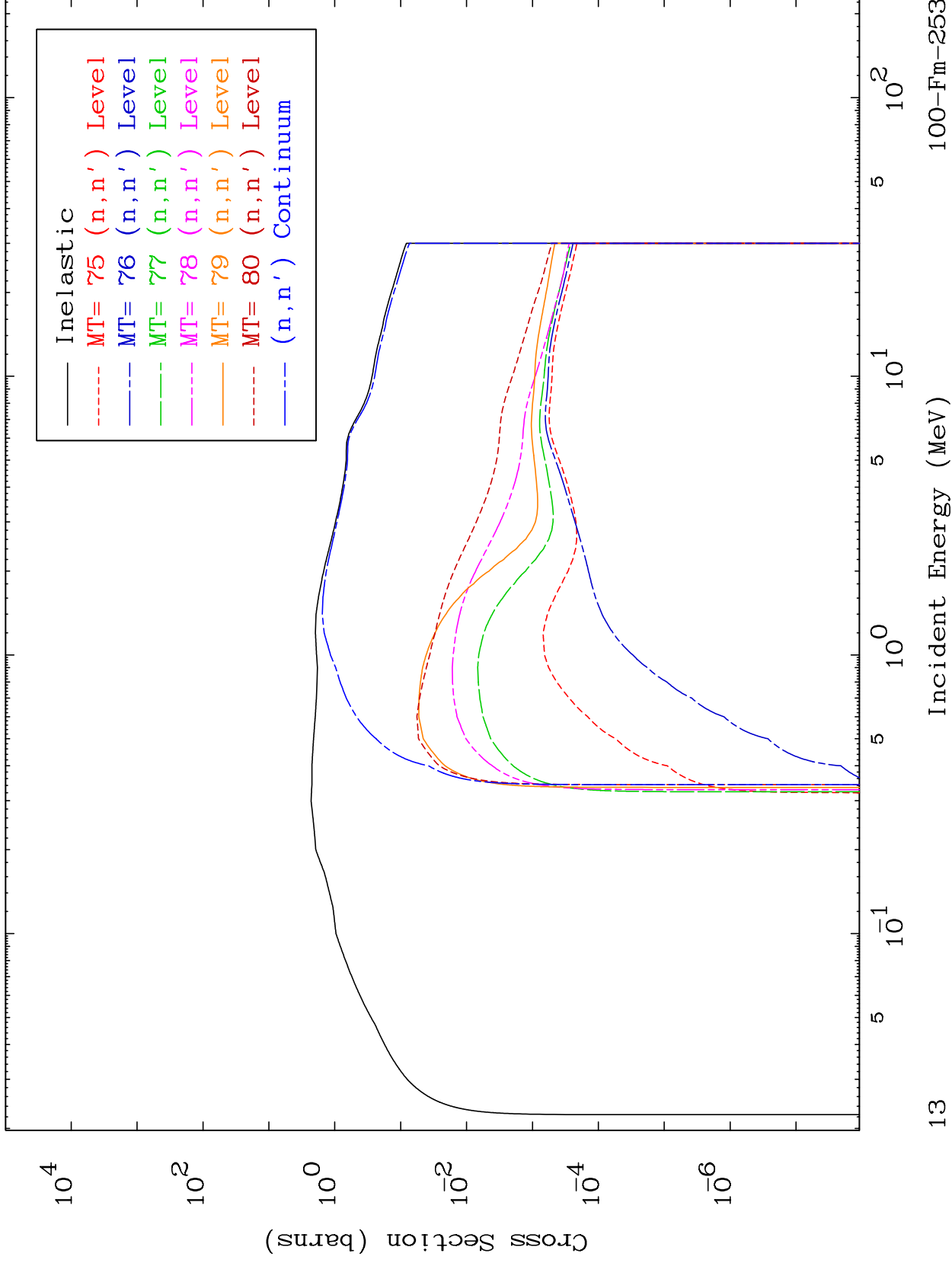
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(n,n') Levels
293 Kelvin Cross Sections

100-Fm-253



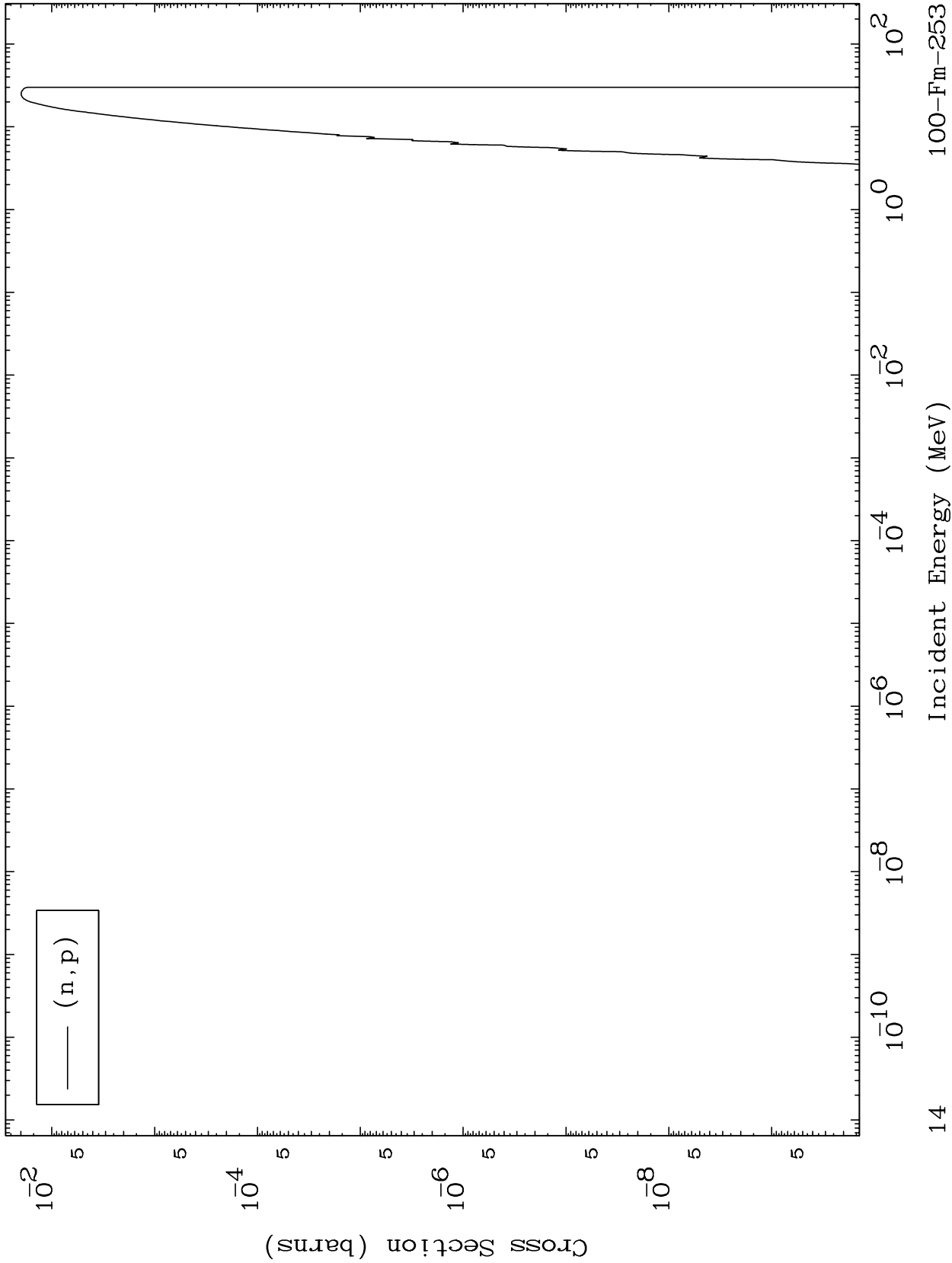
13

100-Fm-253

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(n,p) Levels
293 Kelvin Cross Sections

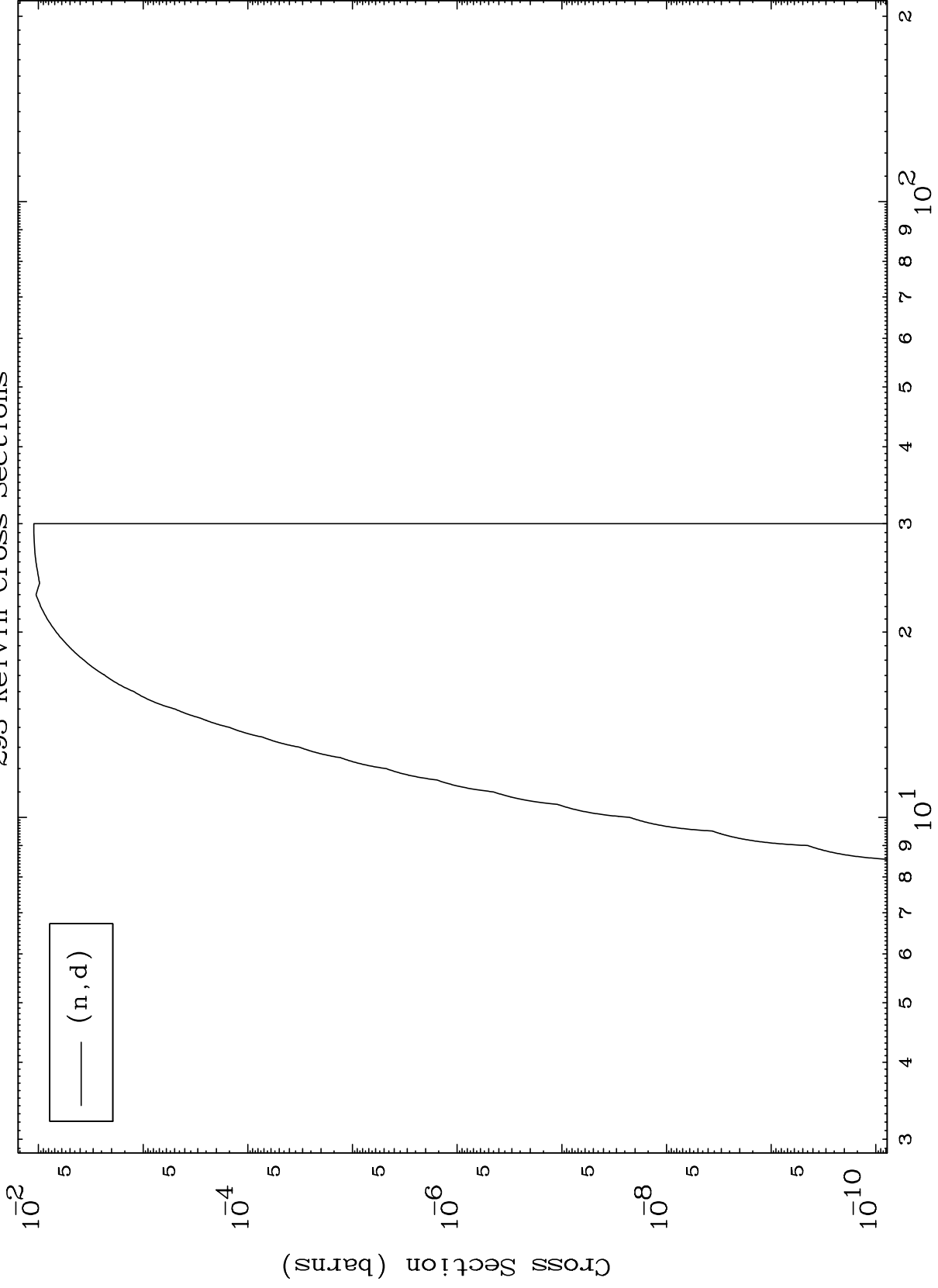
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MAT 9934

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-253



15

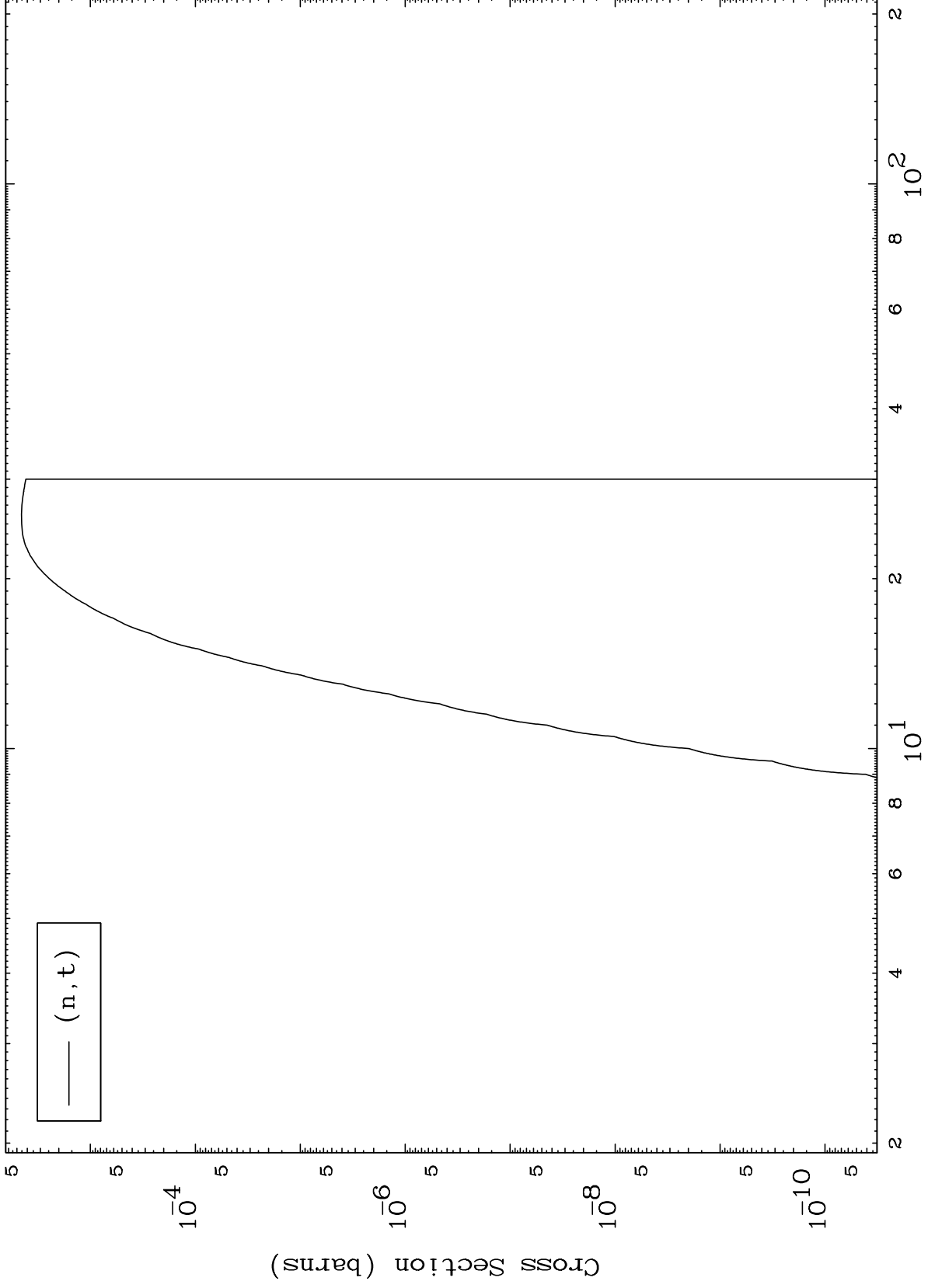
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-253



16

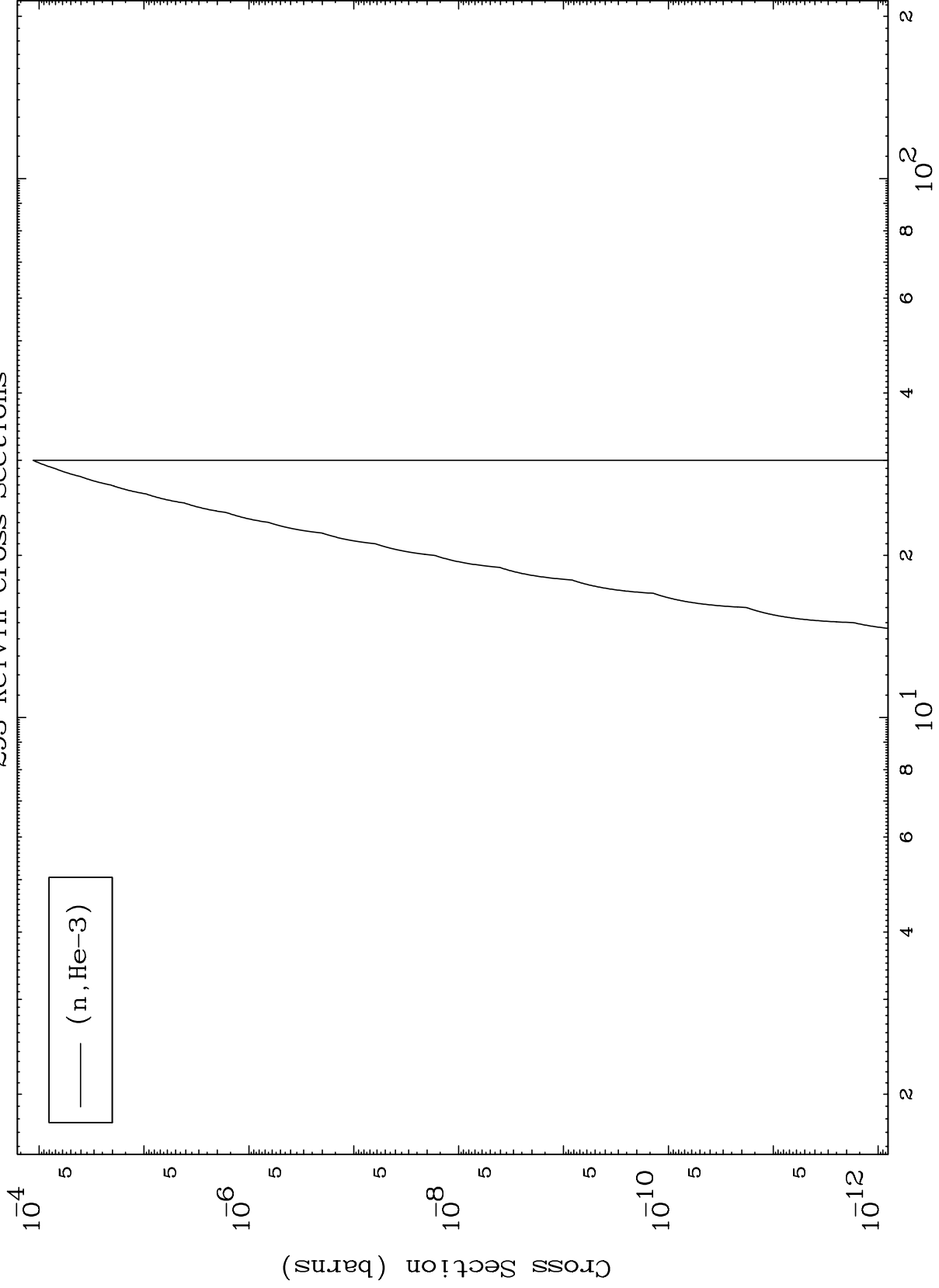
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,He3) Levels
293 Kelvin Cross Sections

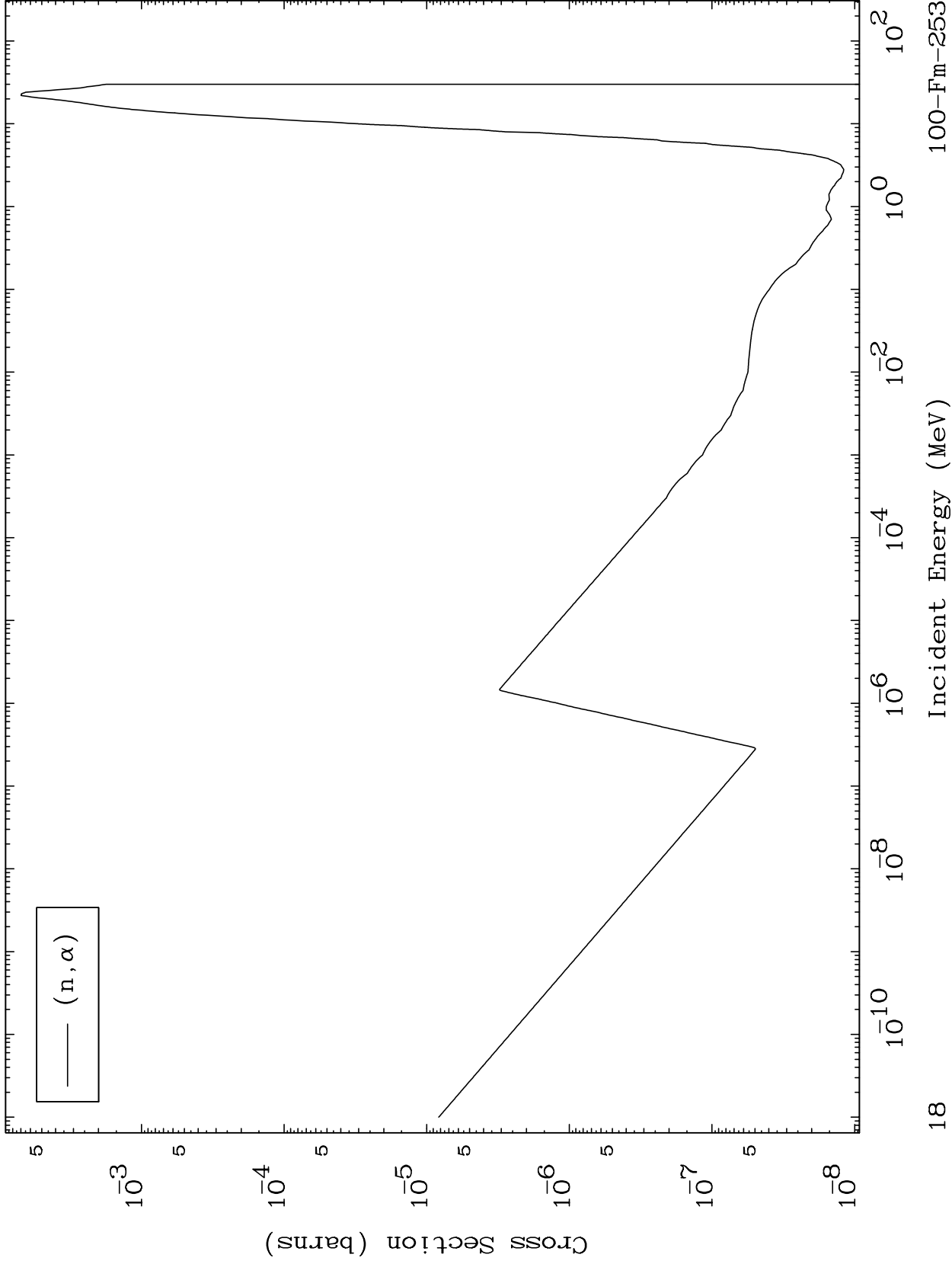
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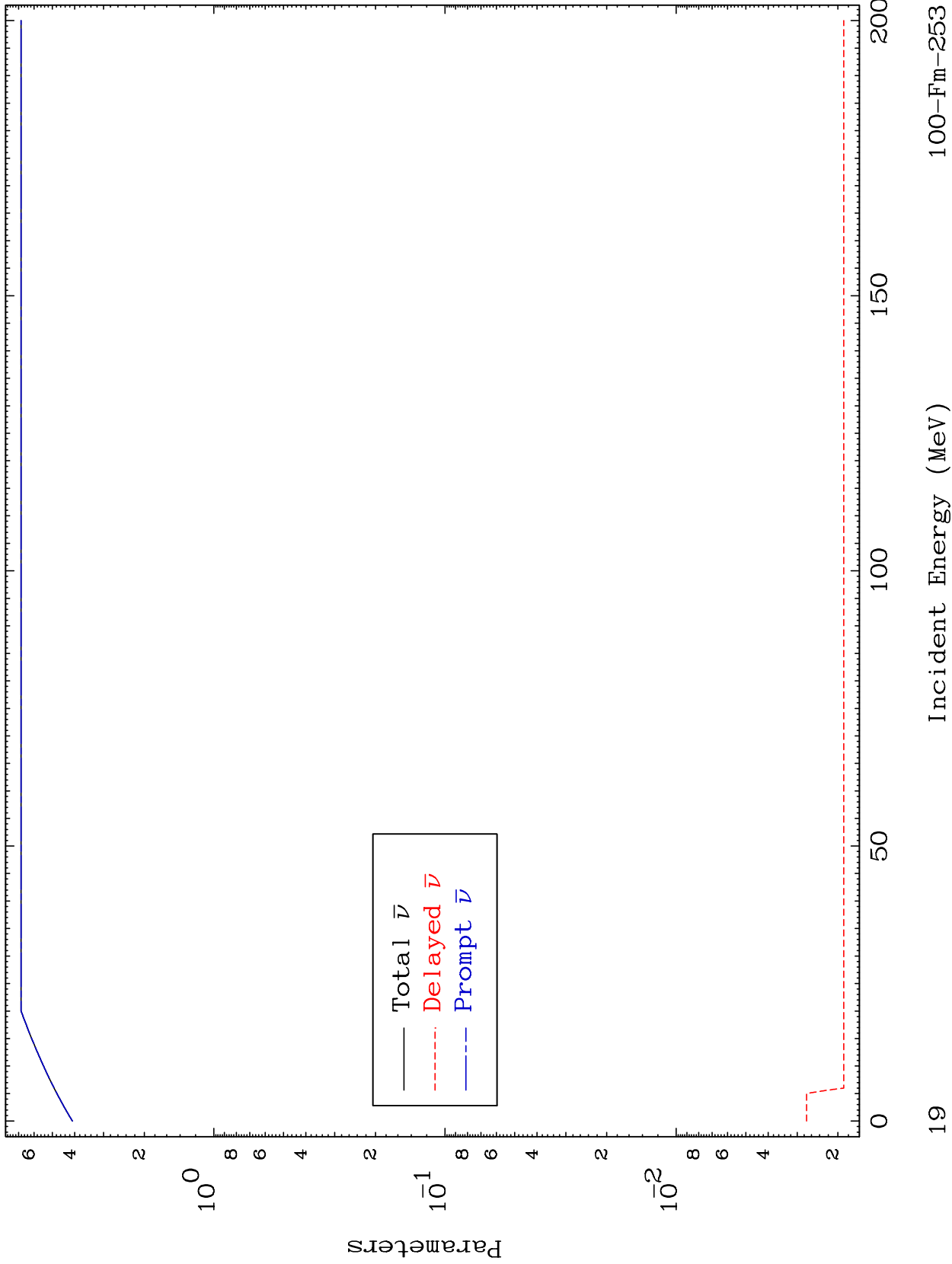


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(n, α) Levels
293 Kelvin Cross Sections

100-Fm-253

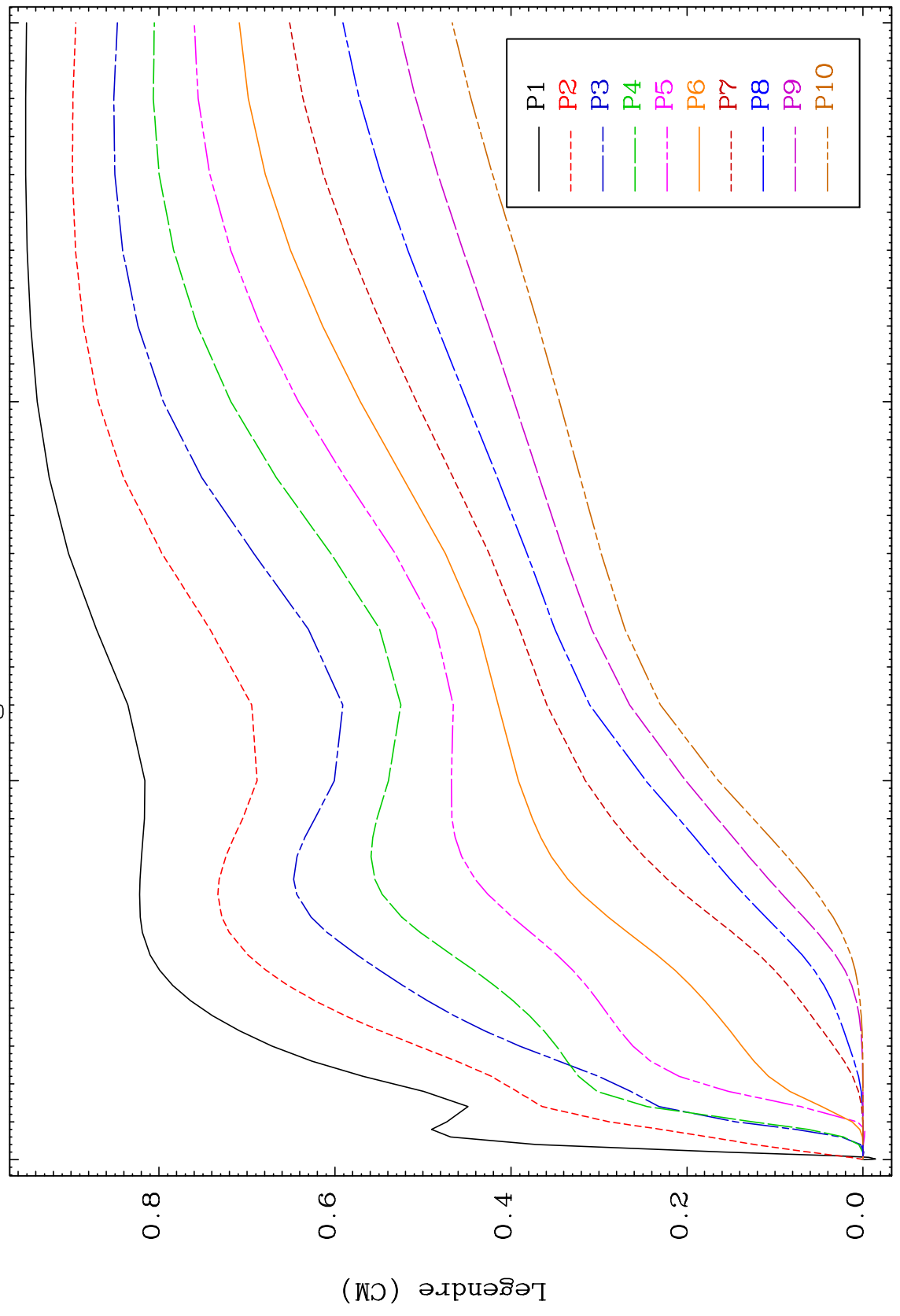




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Elastic Legendre Coefficients

100-Fm-253



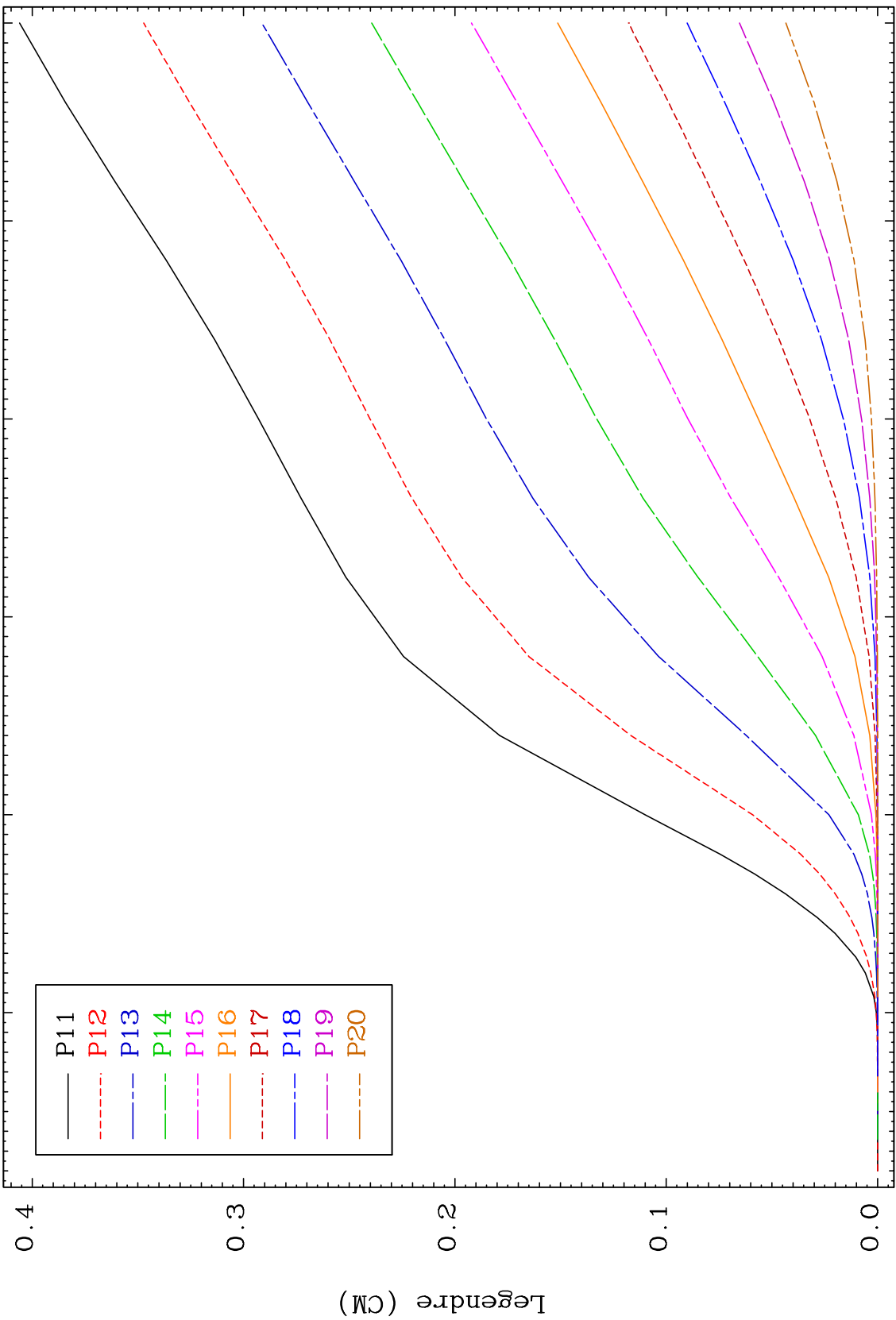
Incident Energy (MeV)

100-Fm-253

MAT 9934

Elastic Legendre Coefficients

100-Fm-253



21

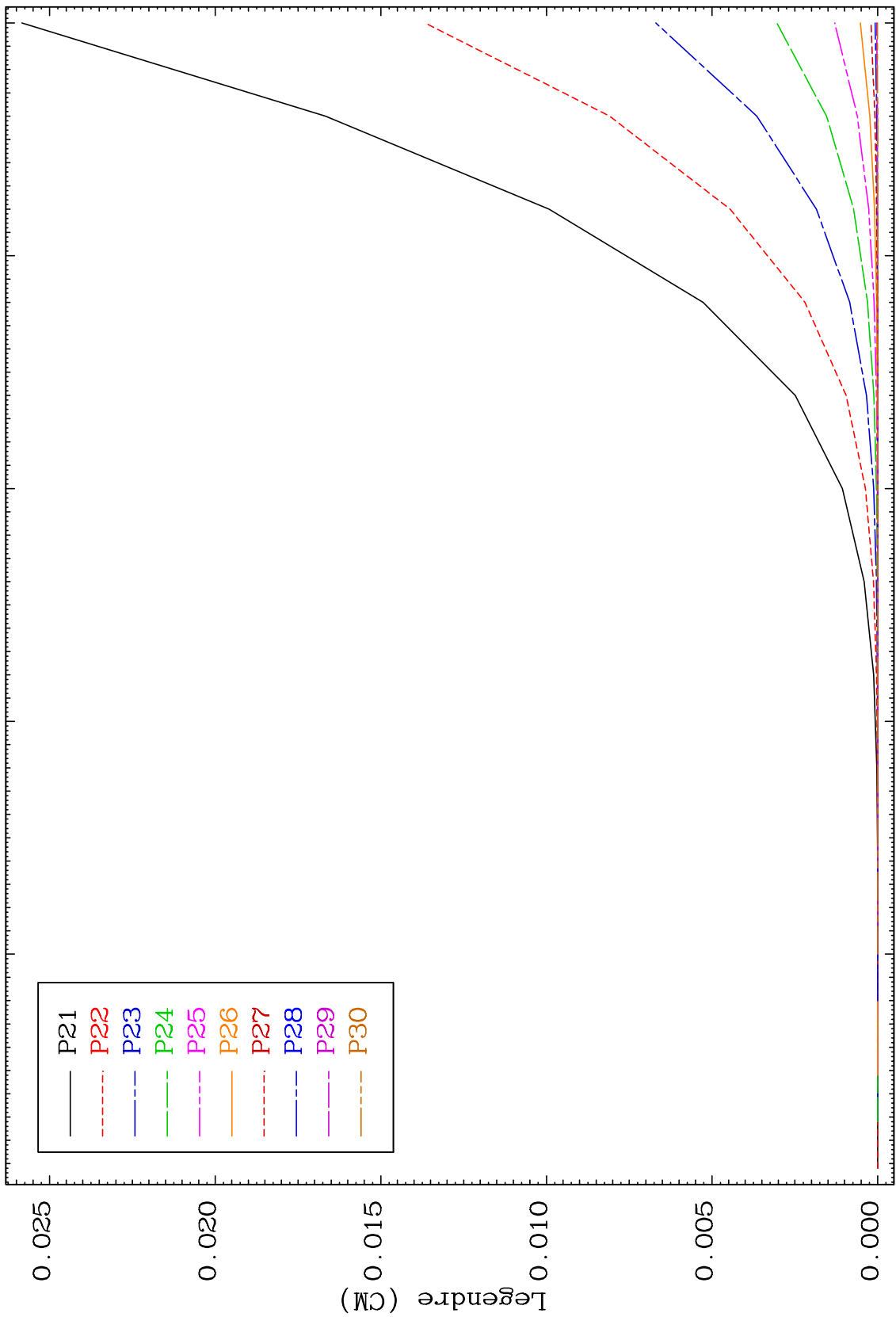
Incident Energy (MeV)

100-Fm-253

MAT 9934

Elastic Legendre Coefficients

100-Fm-253



22

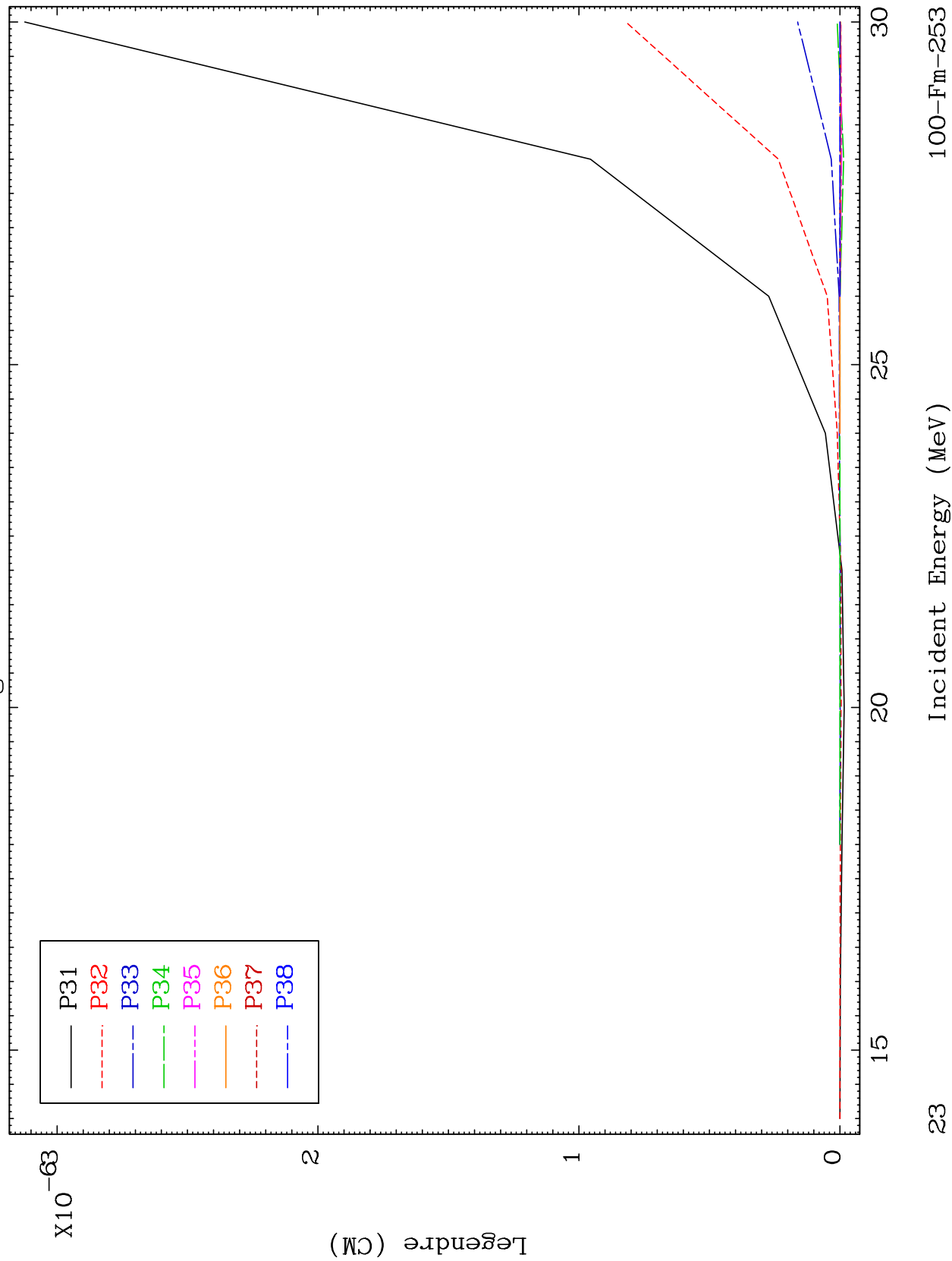
Incident Energy (MeV)

100-Fm-253

MAT 9934

Elastic
Legendre Coefficients

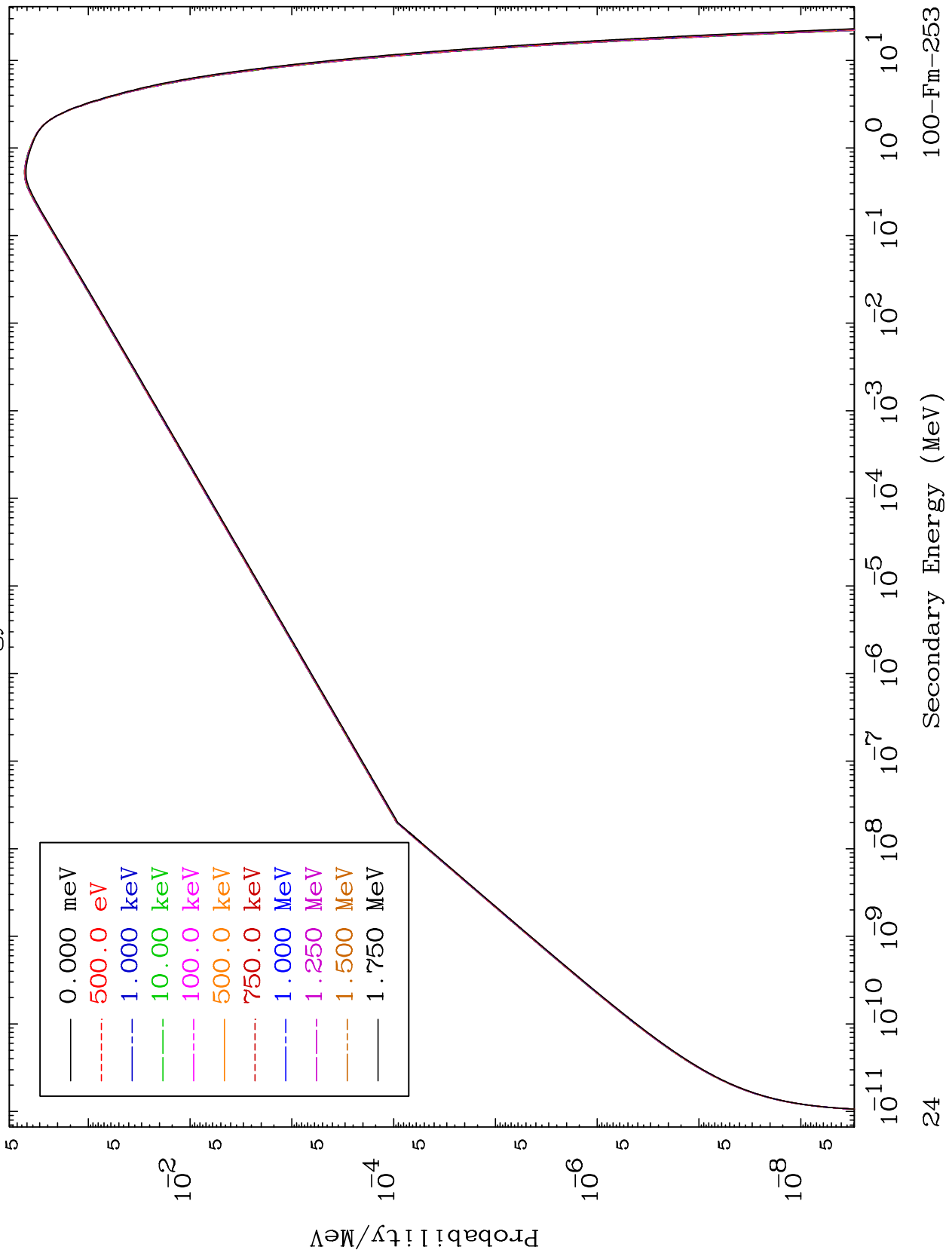
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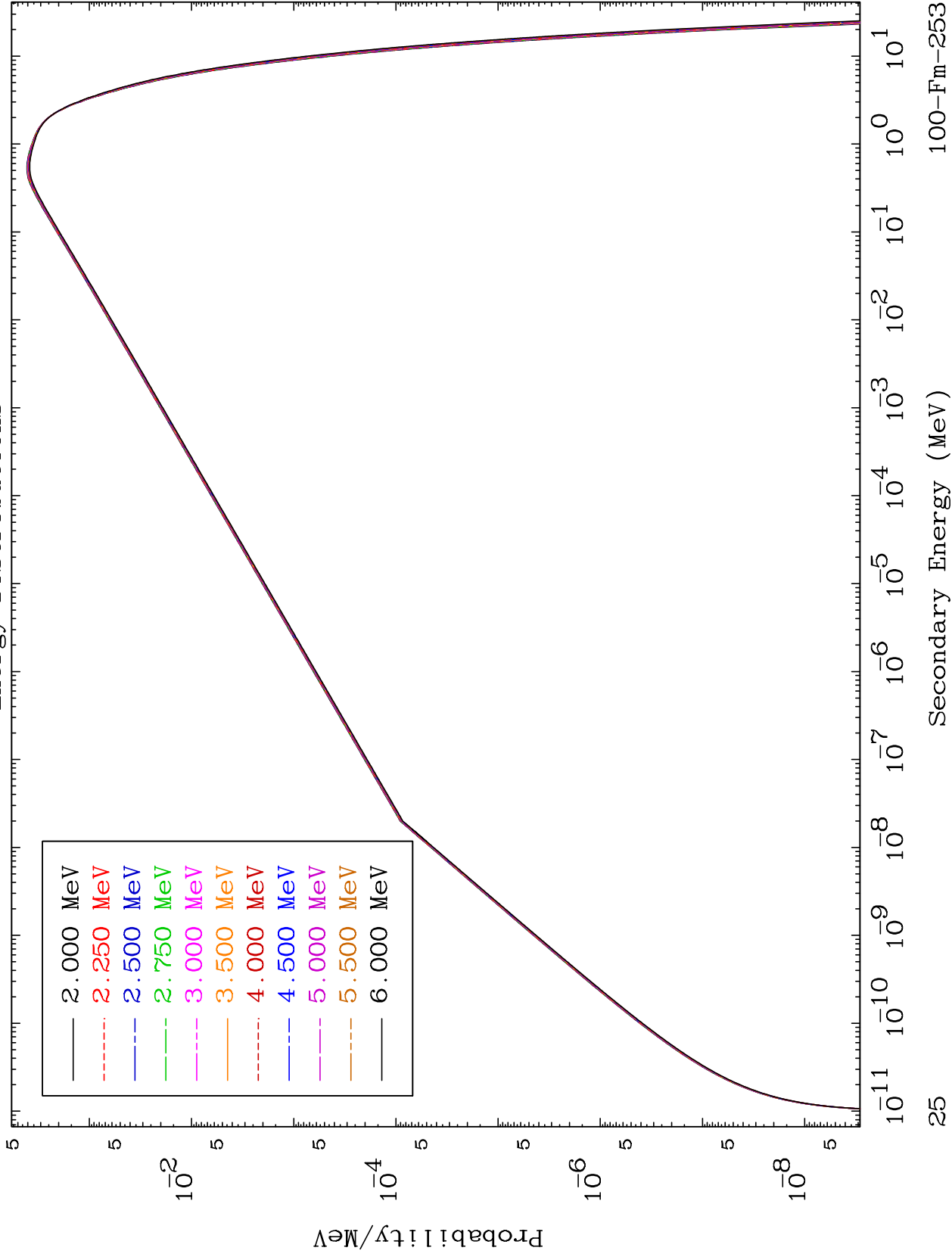


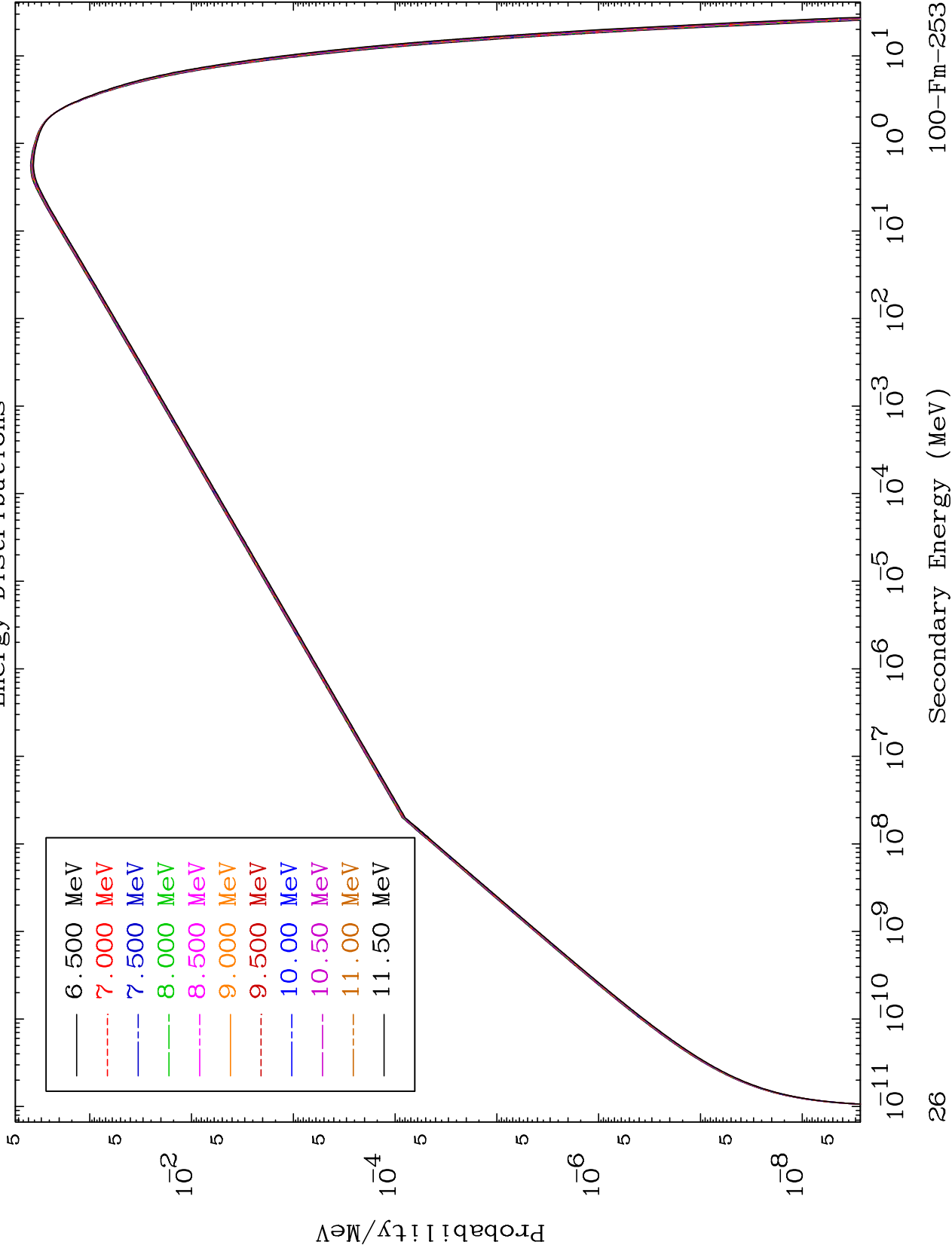
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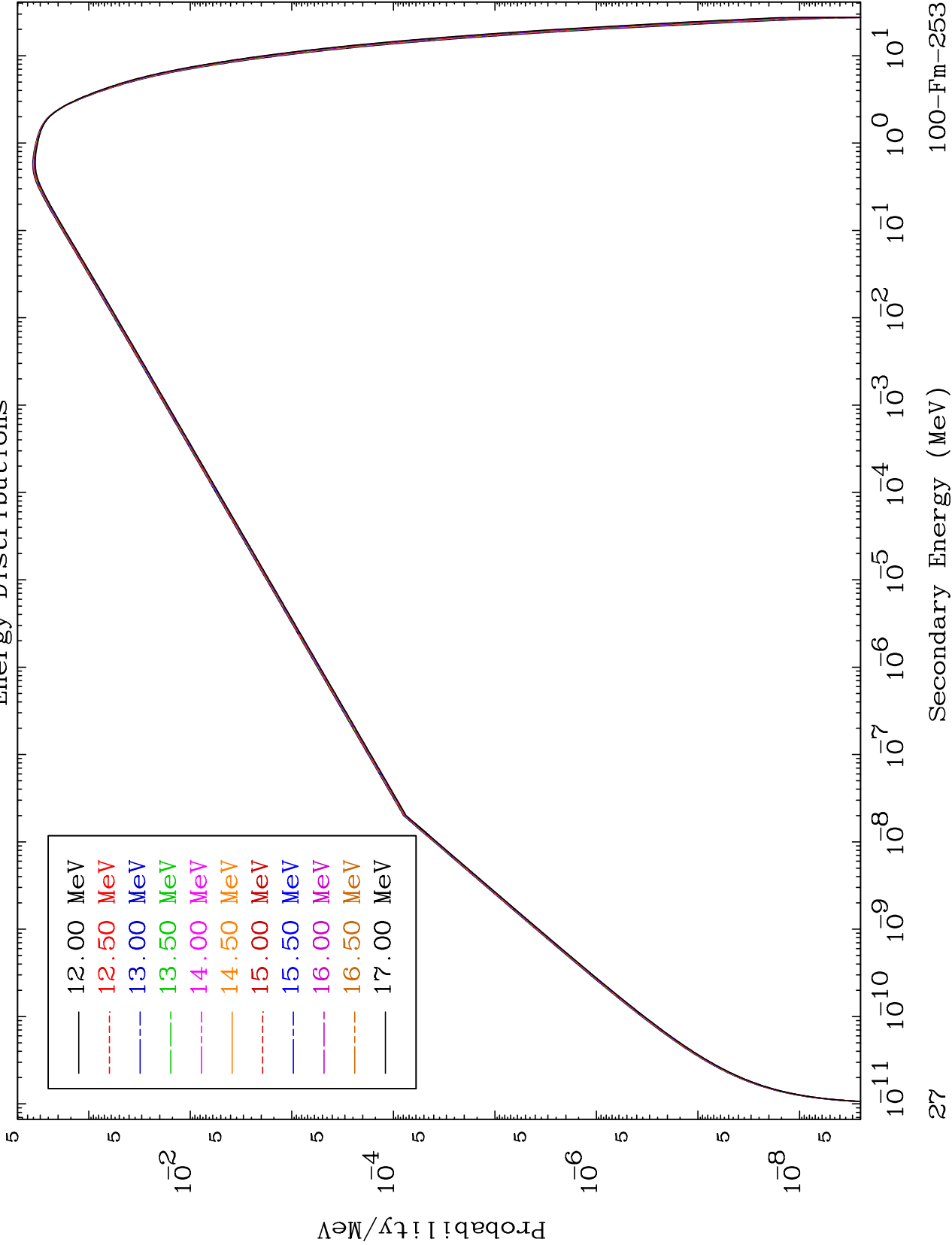
Incident Energy (MeV)

100-Fm-253





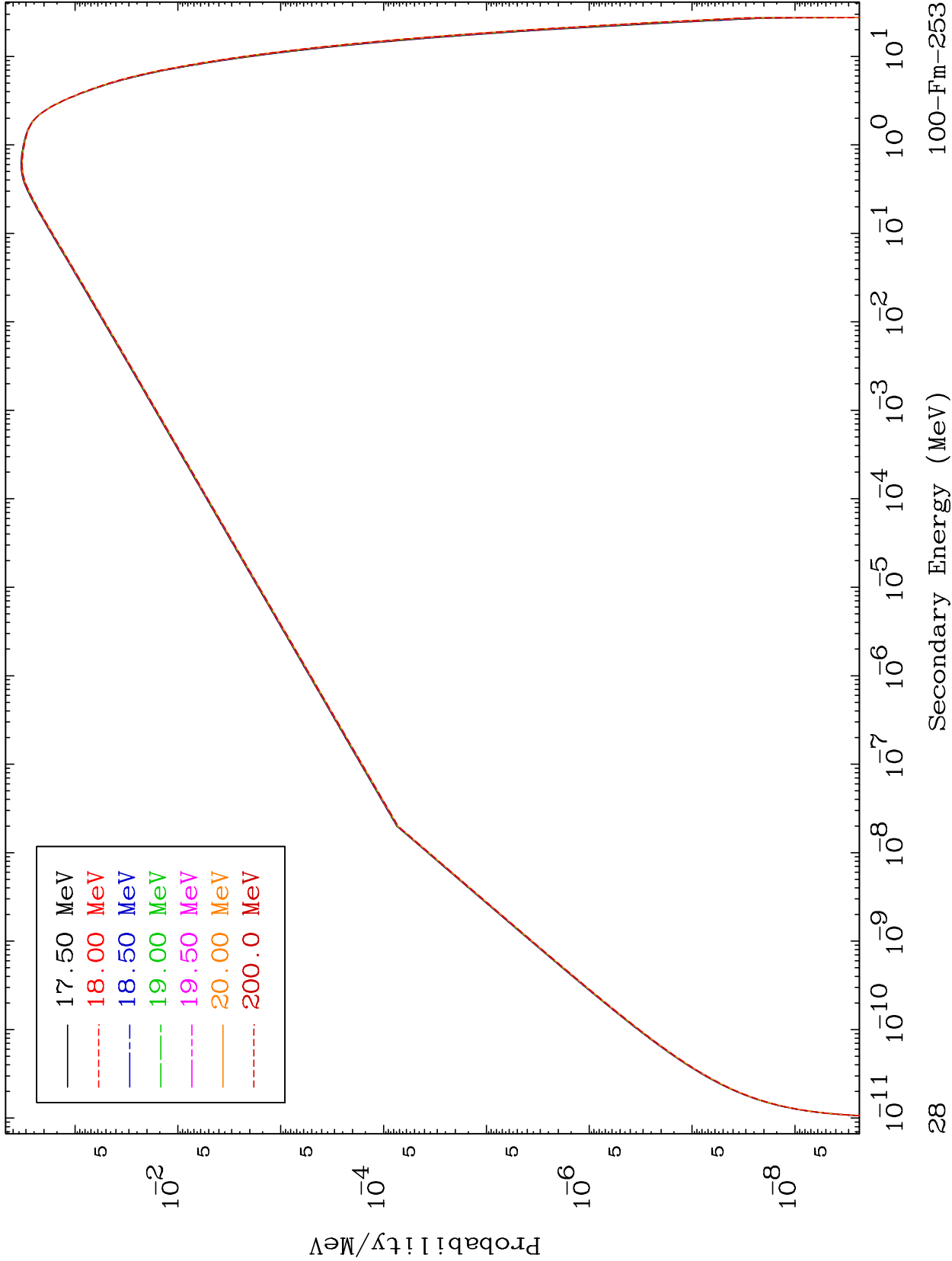




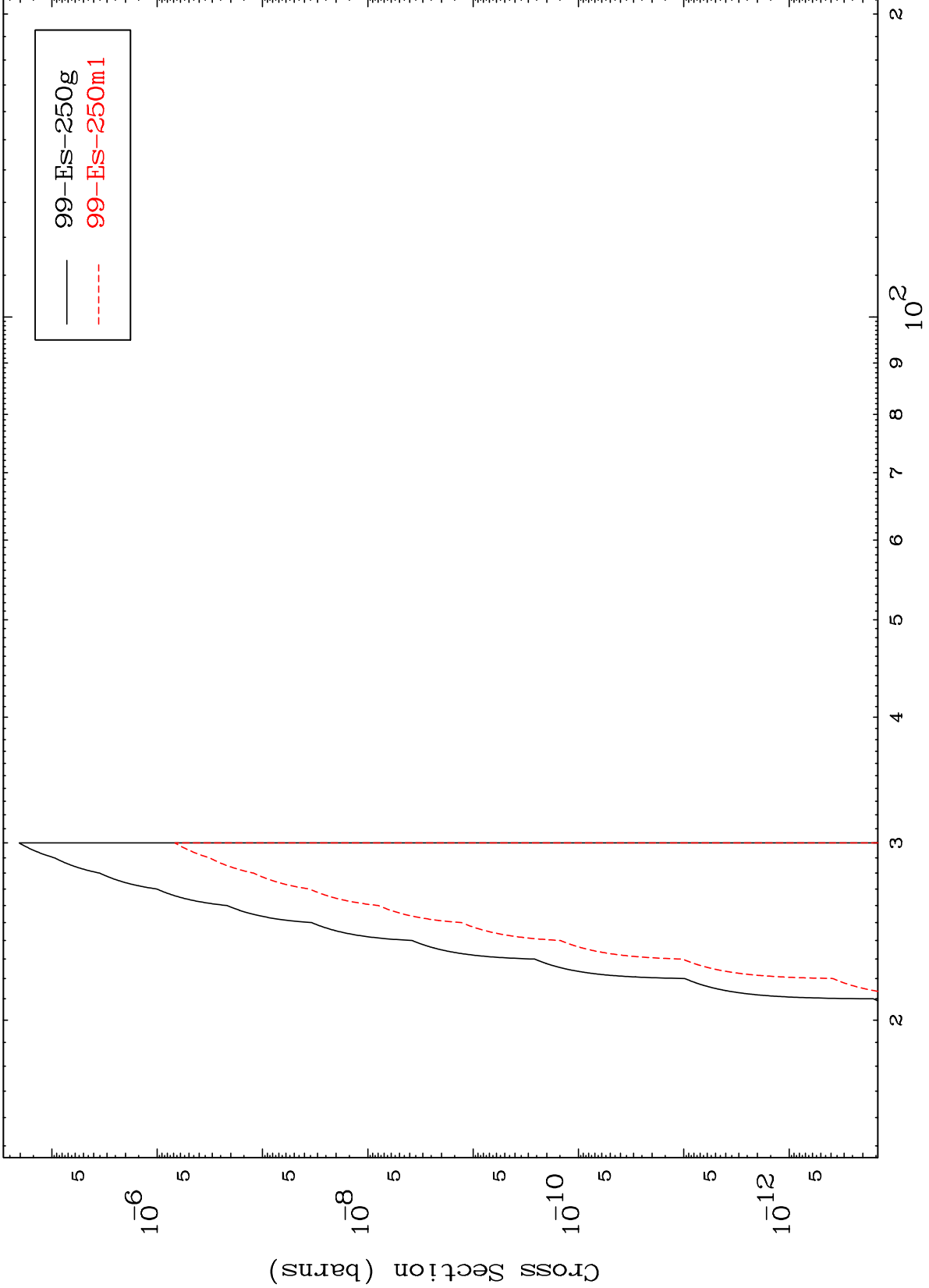
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Fission Energy Distributions

100-Fm-253



Radionuclide Production Cross Section



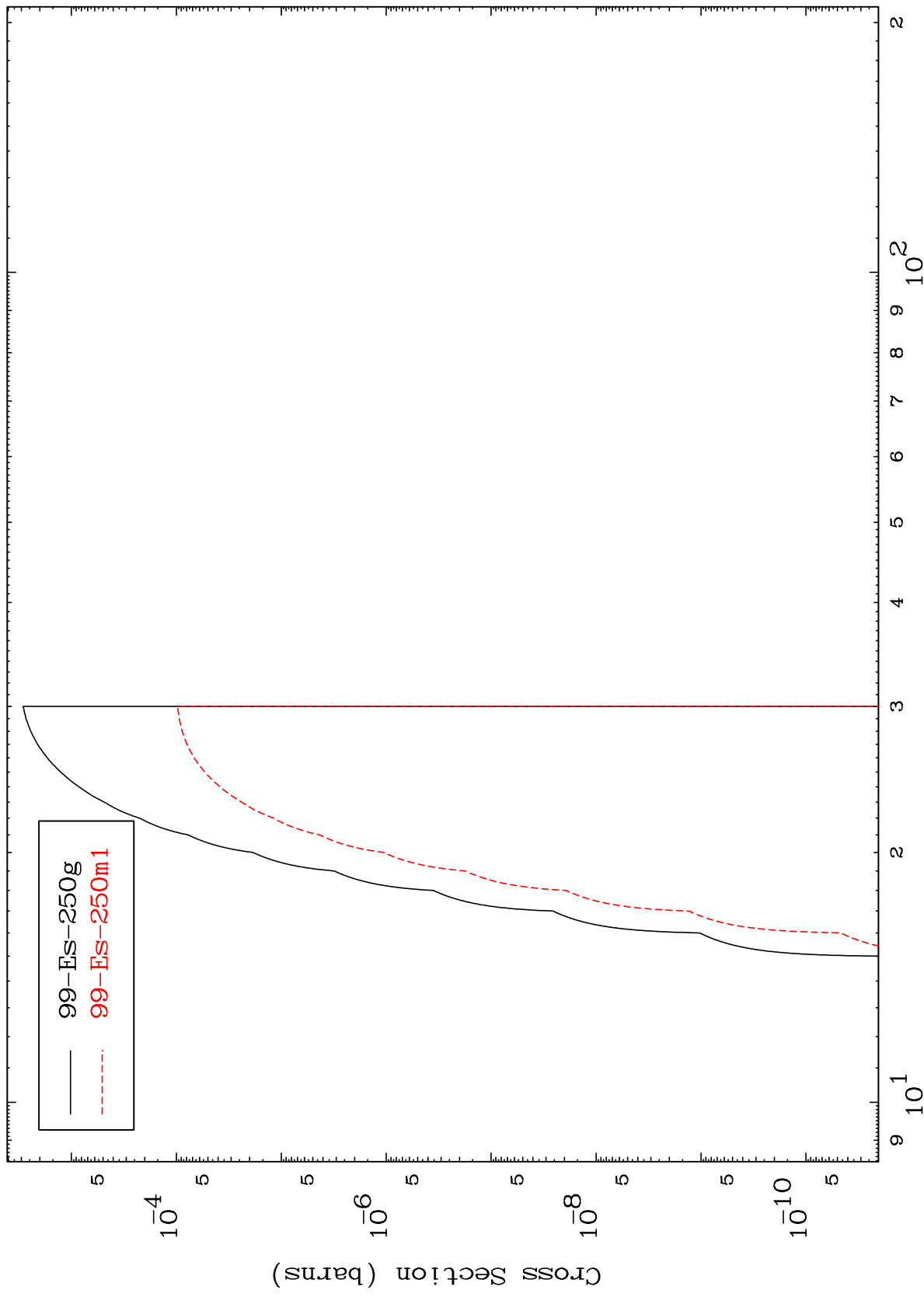
99-Es-250g
99-Es-250m1

MAT 9934

(n,n') t

100-Fm-253

Radionuclide Production Cross Section

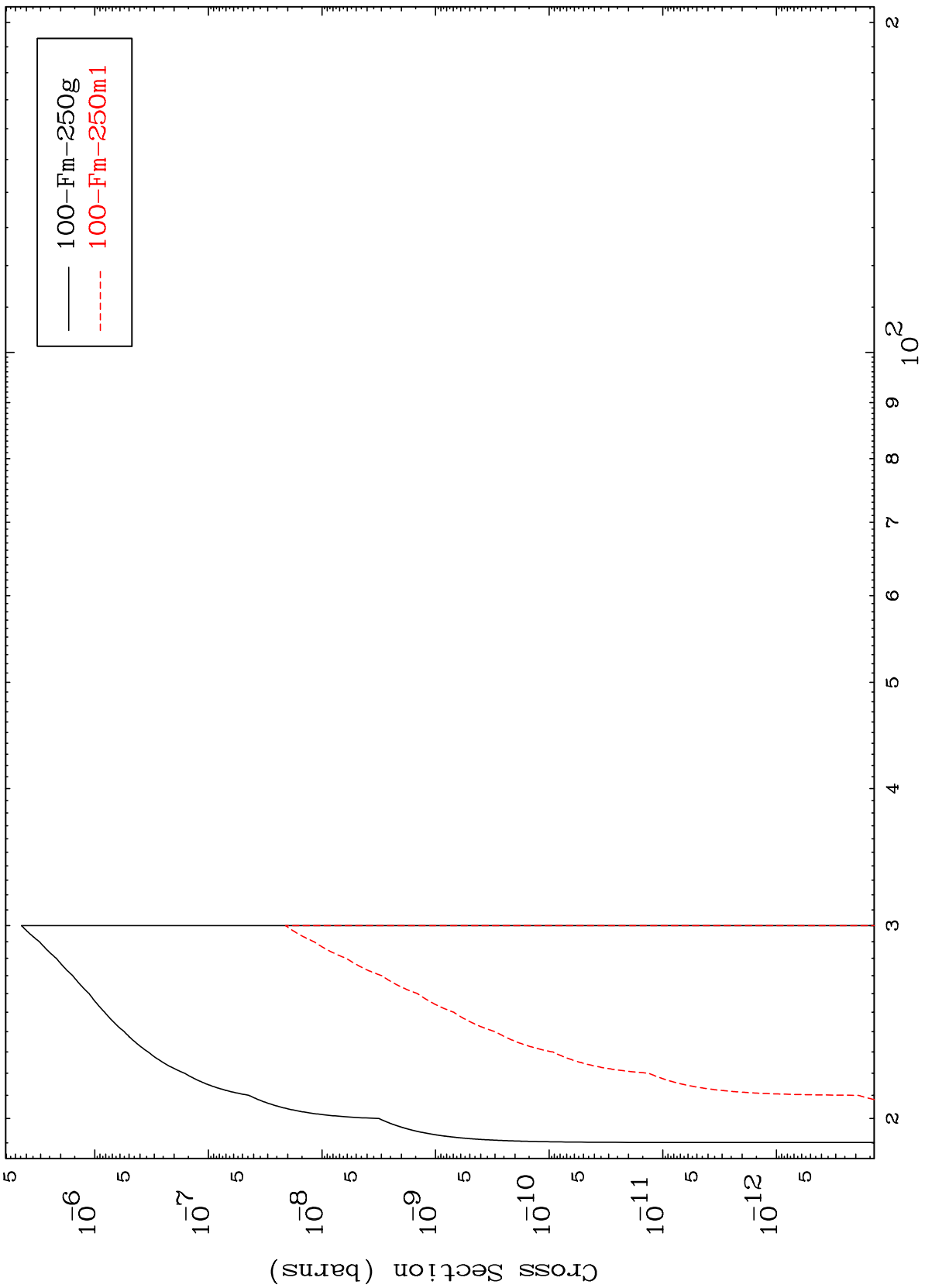


Incident Energy (MeV)

100-Fm-253

30

(n,4n)
Radionuclide Production Cross Section

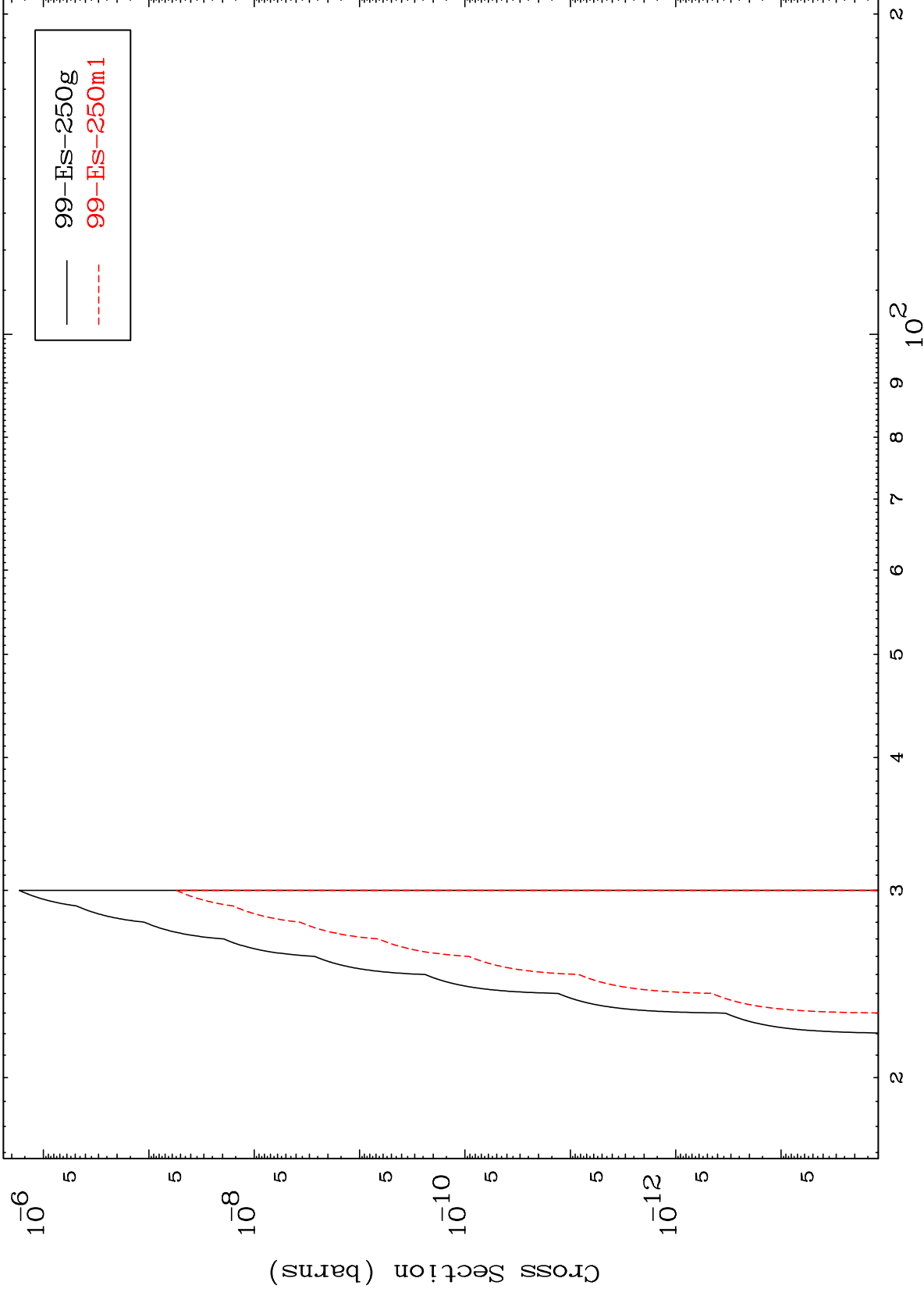


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(n,3n) p

100-Fm-253

Radionuclide Production Cross Section



99-Es-250g
99-Es-250m1

32

Incident Energy (MeV)

100-Fm-253

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(n,n') p α

100-Fm-253

Radionuclide Production Cross Section

