

Program EVALPLOT
(Version 2021-1)

by

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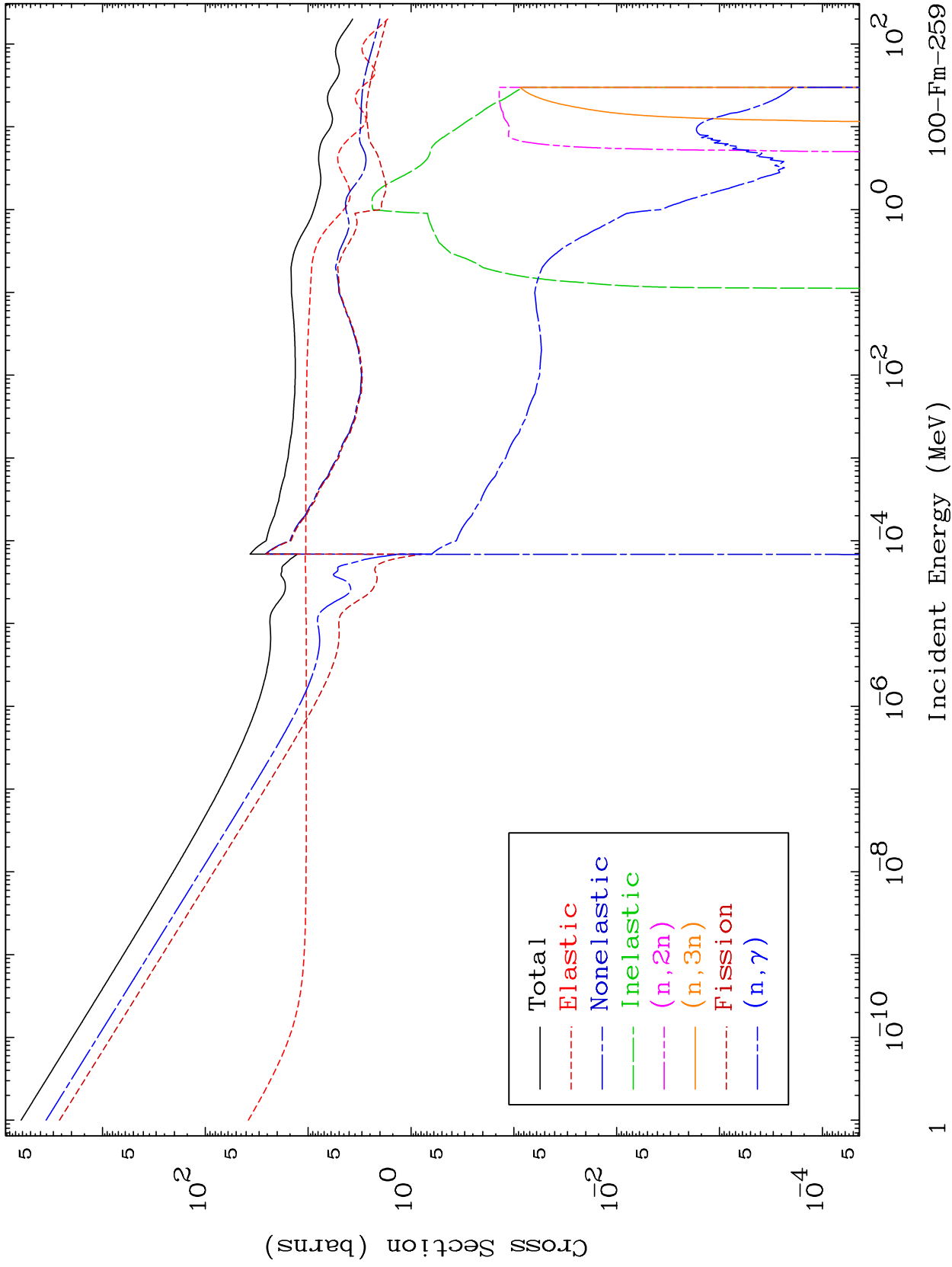
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Press Mouse Button to Start

MAT 9940

Neutron Major
293 Kelvin Cross Sections

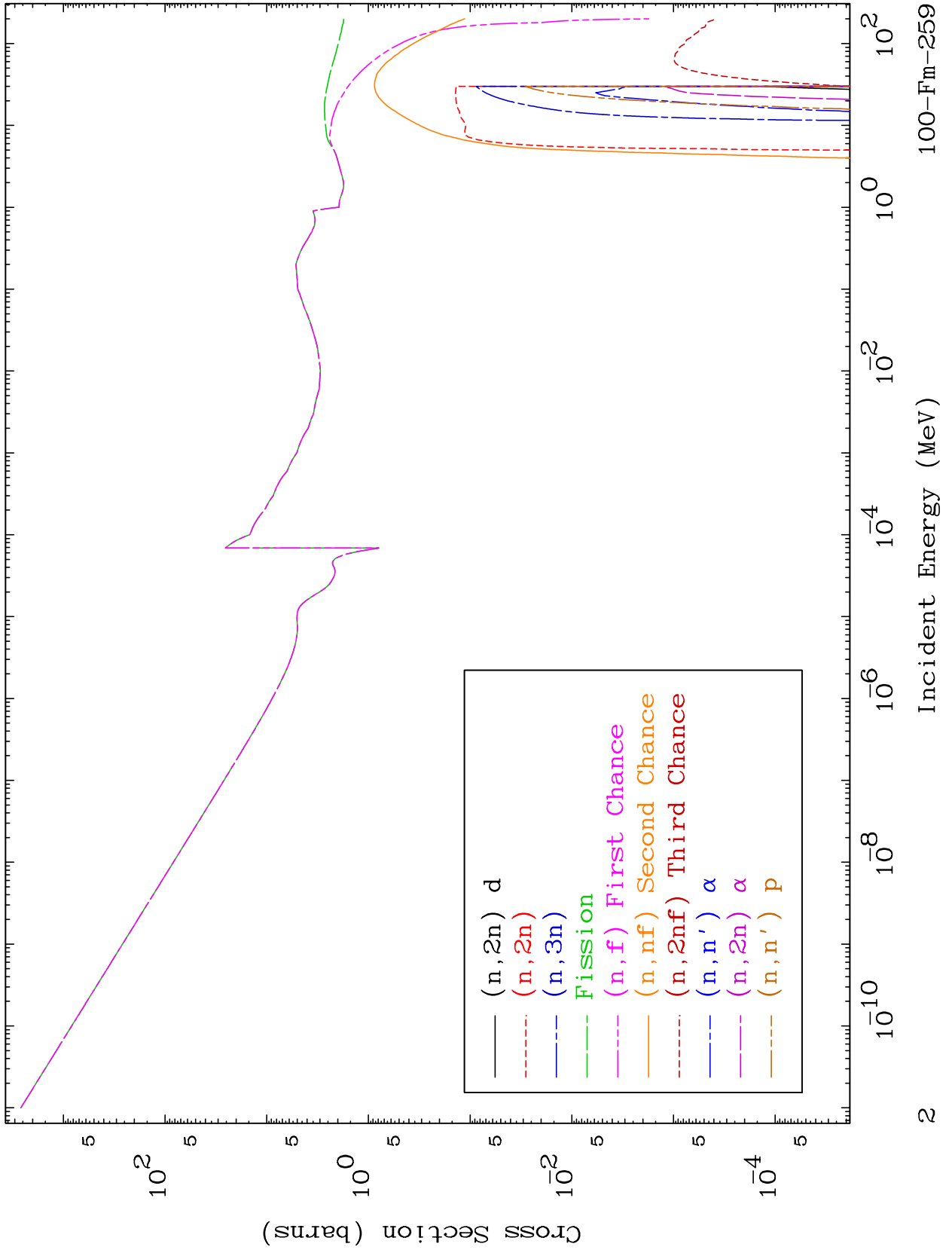
100-Fm-259



MAT 9940

Neutron Absorption
293 Kelvin Cross Sections

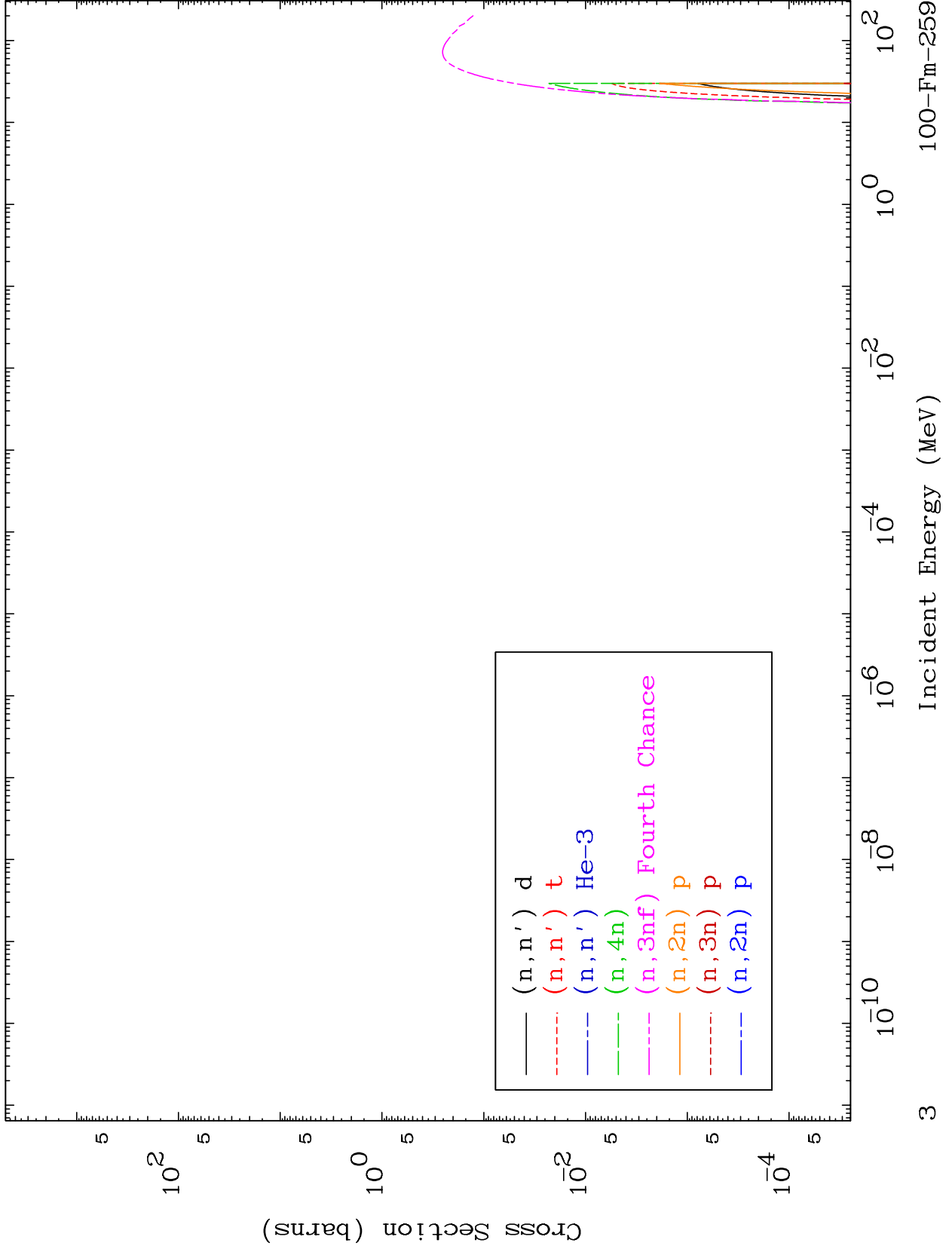
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MAT 9940

Neutron Absorption
293 Kelvin Cross Sections

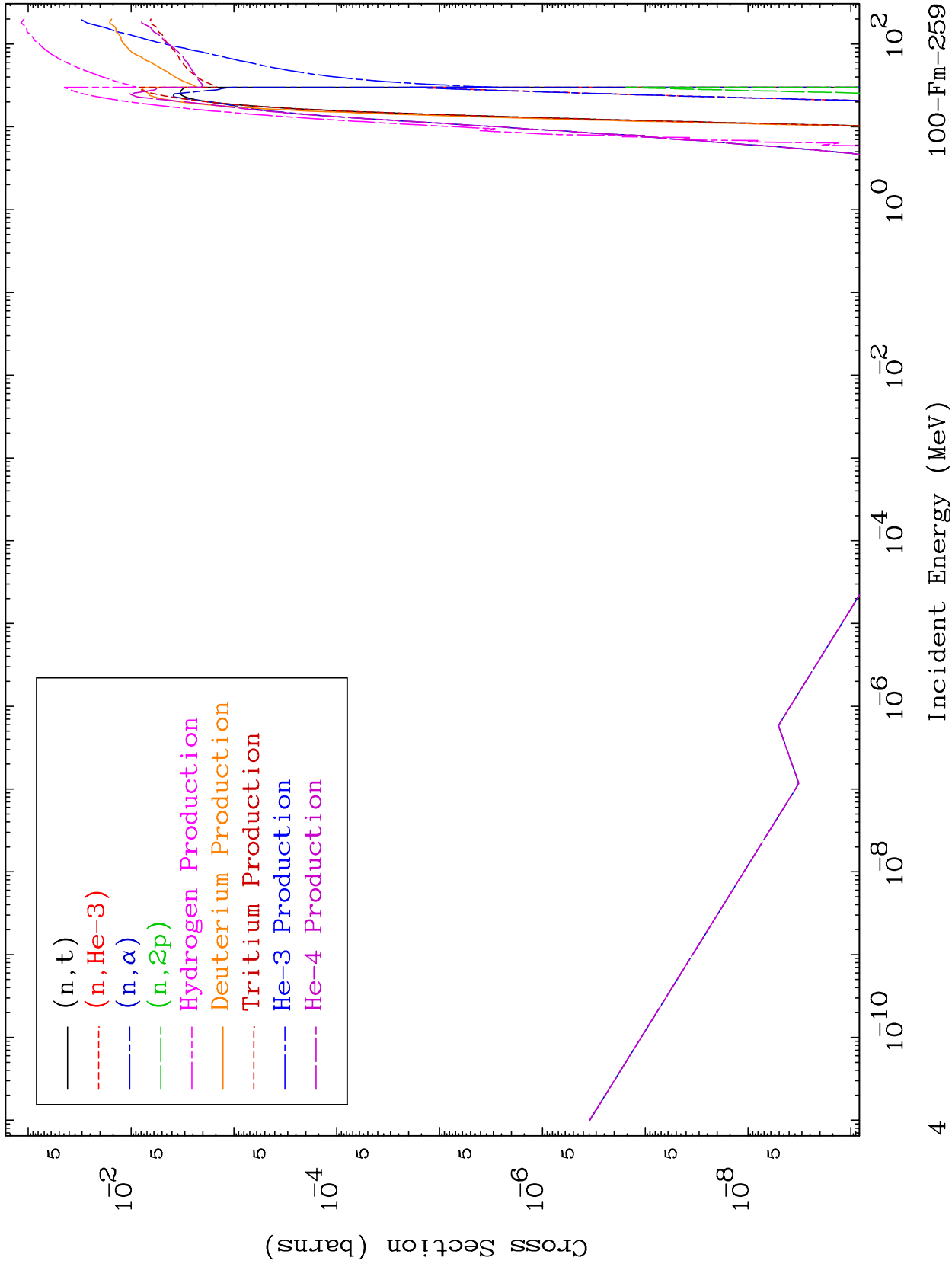
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Neutron Absorption
293 Kelvin Cross Sections

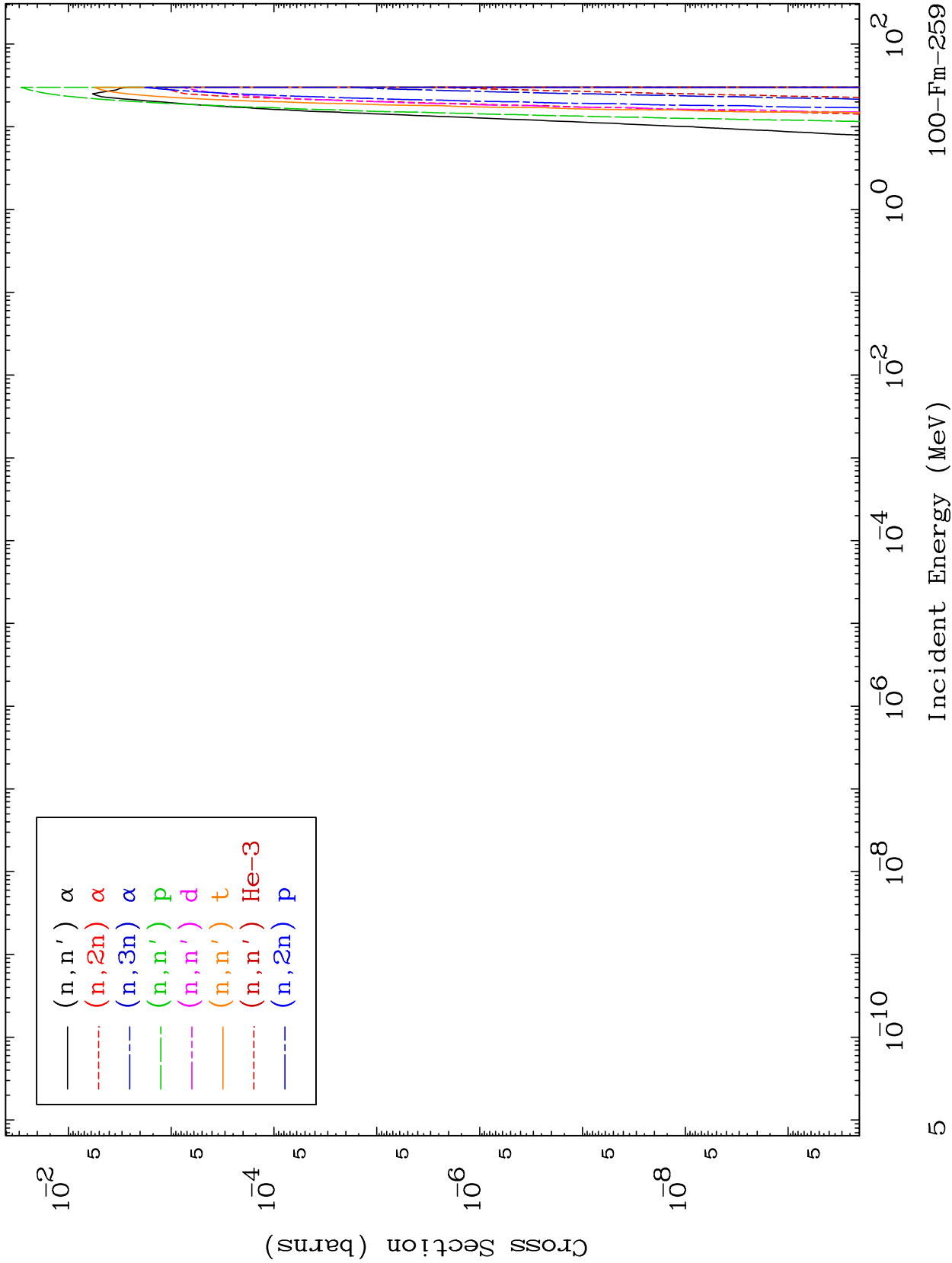
100-Fm-259



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Charged Particle
293 Kelvin Cross Sections

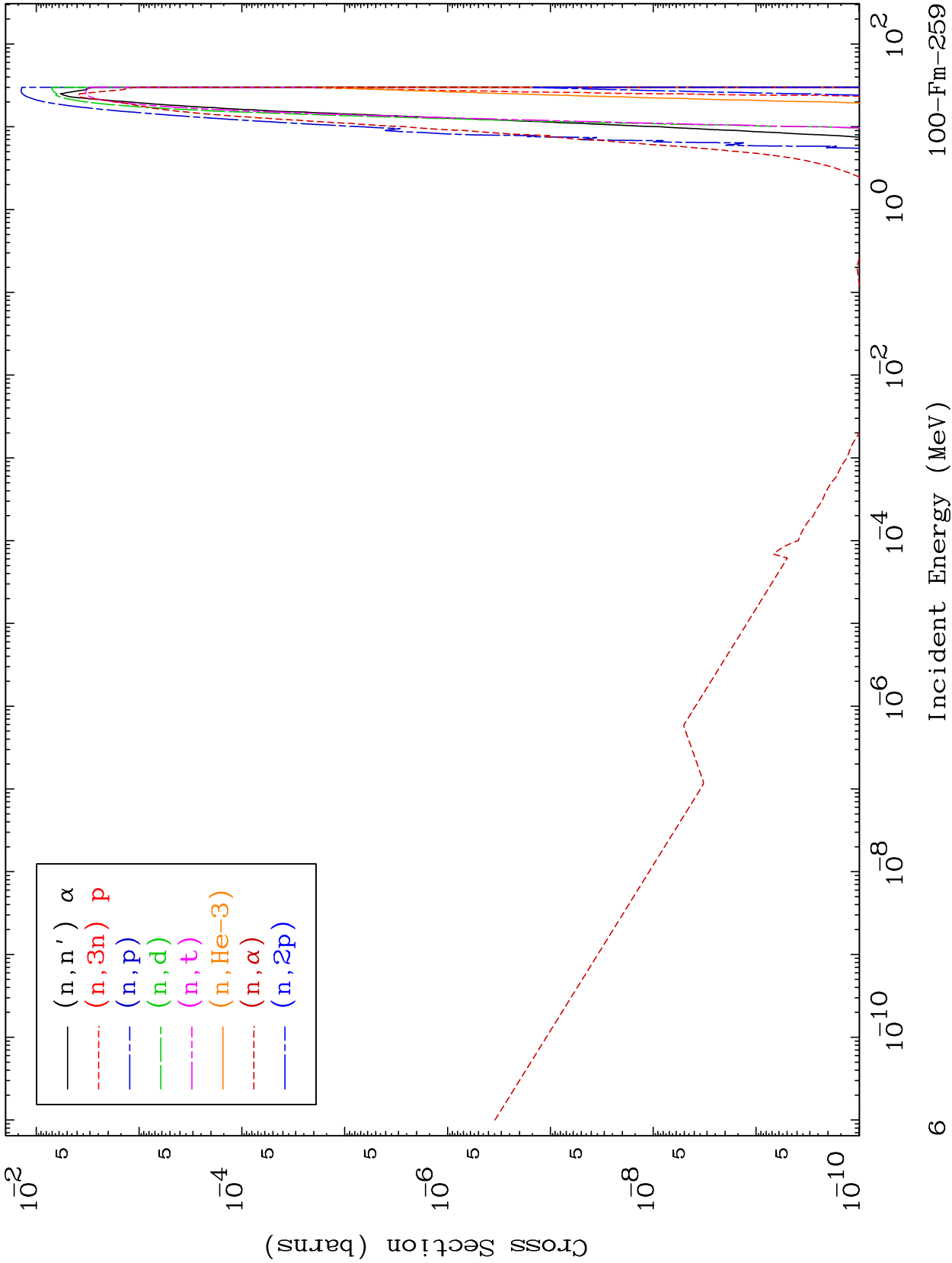
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Charged Particle
293 Kelvin Cross Sections

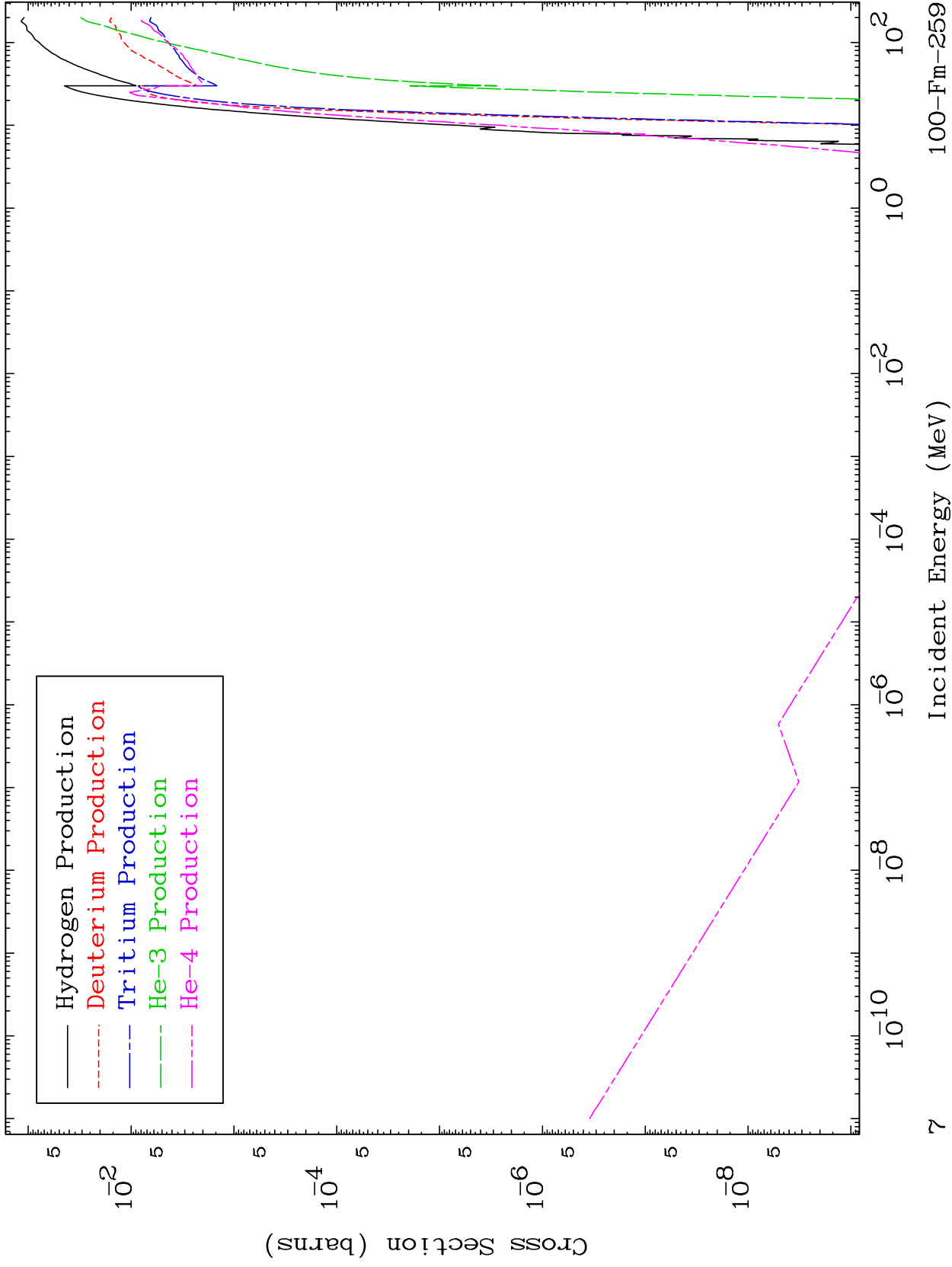
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Particle Production
293 Kelvin Cross Sections

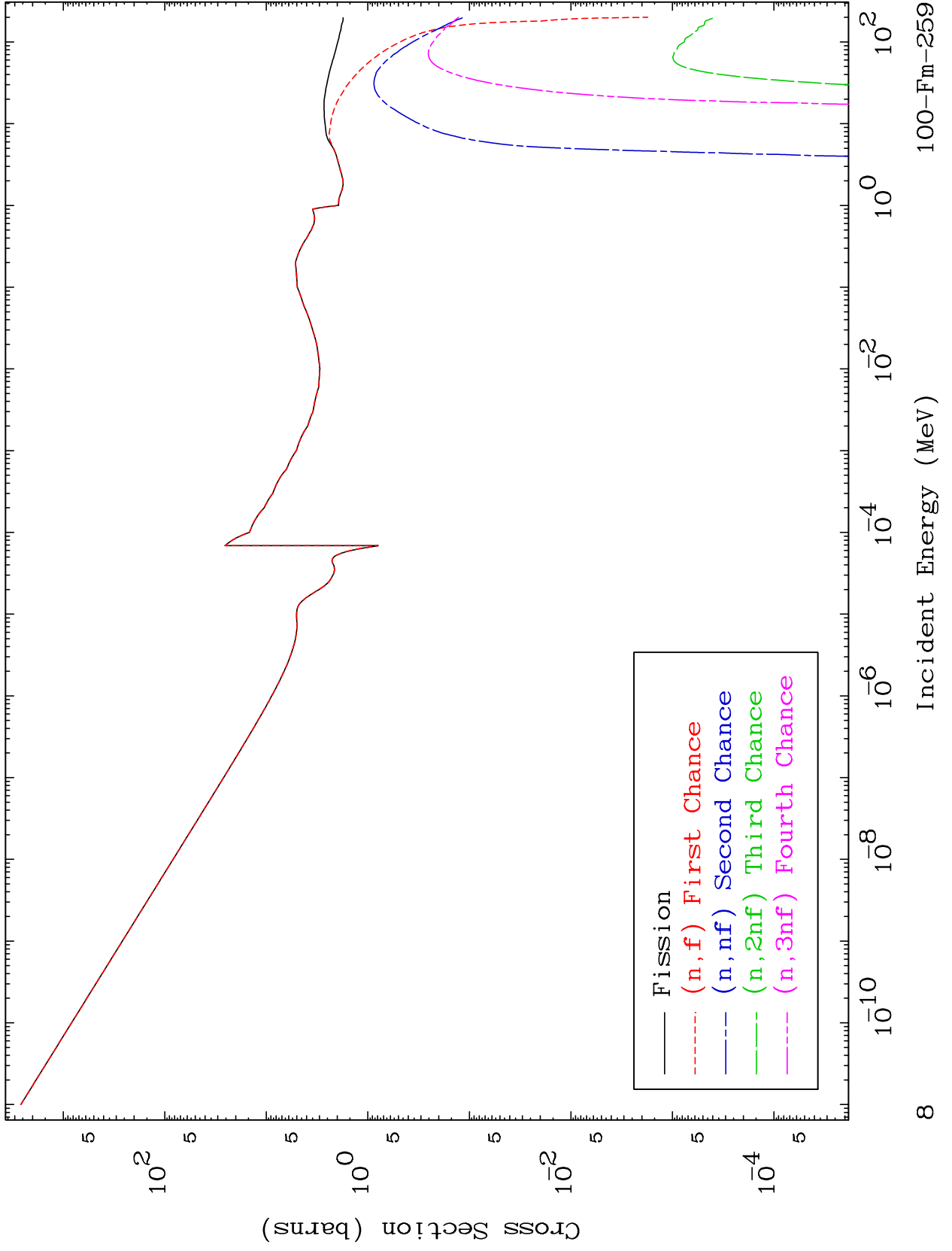
100-Fm-259



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Fission
293 Kelvin Cross Sections

100-Fm-259

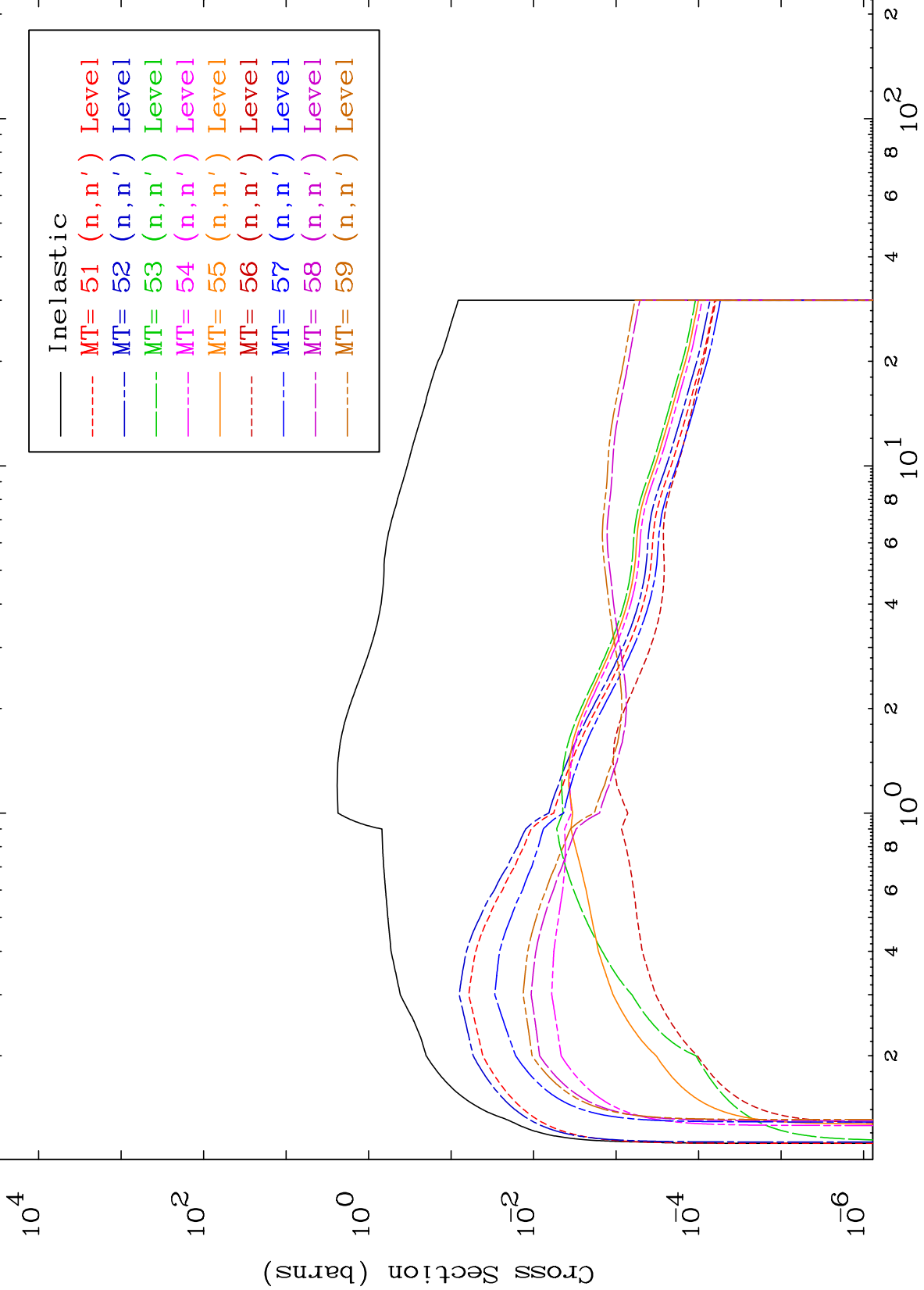


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(n,n') Levels

100-Fm-259

293 Kelvin Cross Sections

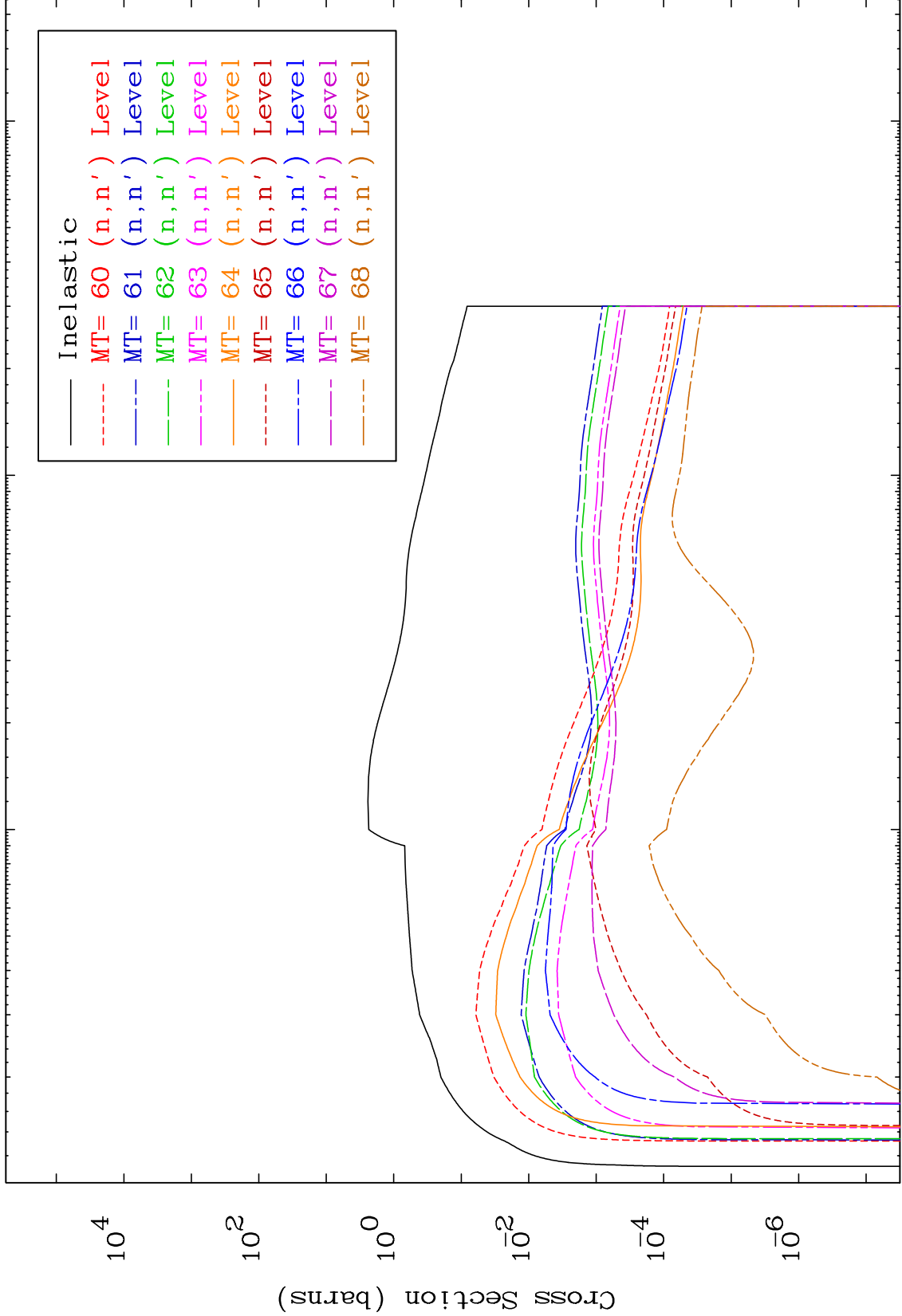


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(n,n') Levels

100-Fm-259

293 Kelvin Cross Sections

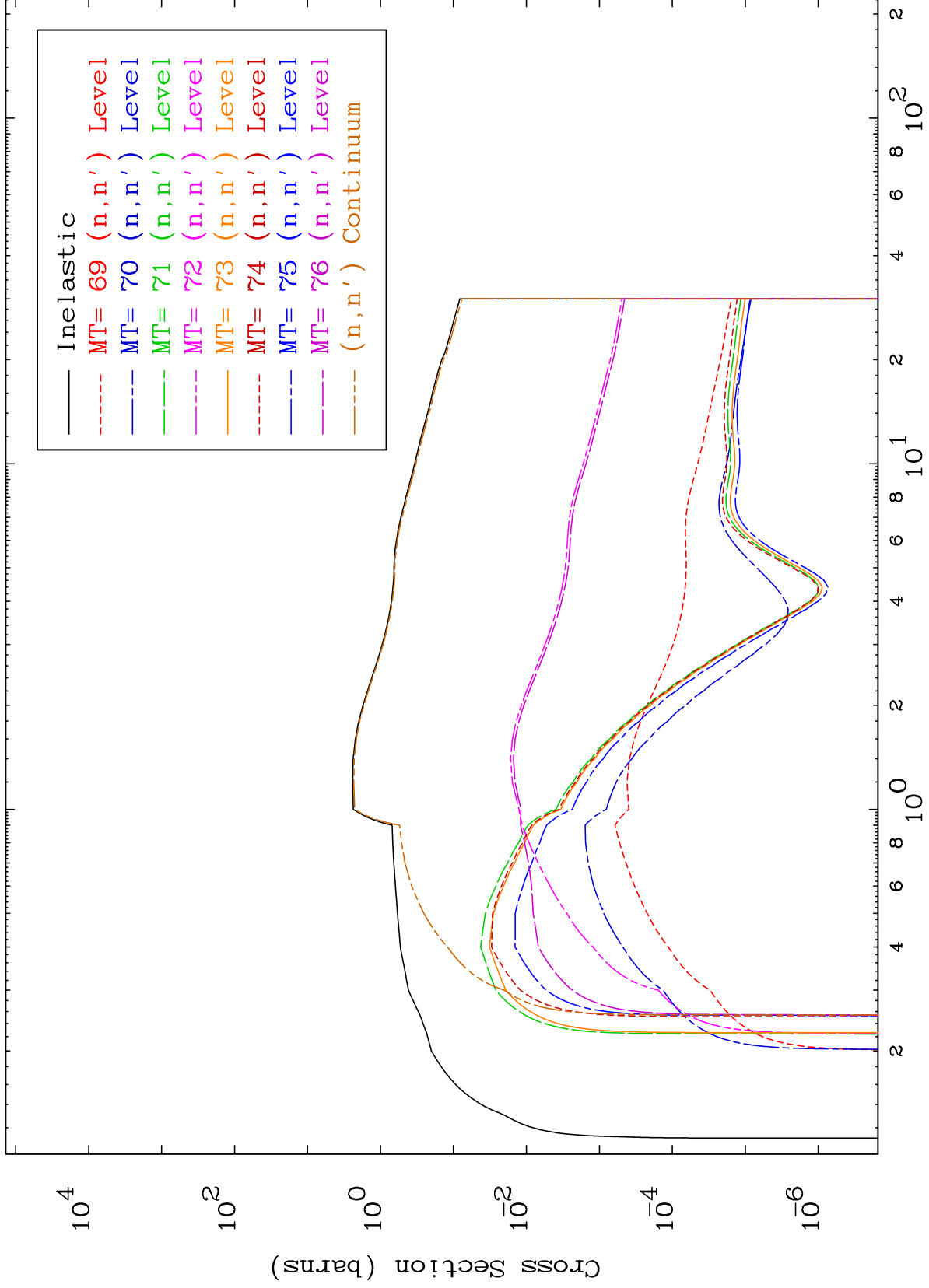


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Incident Energy (MeV)

100-Fm-259

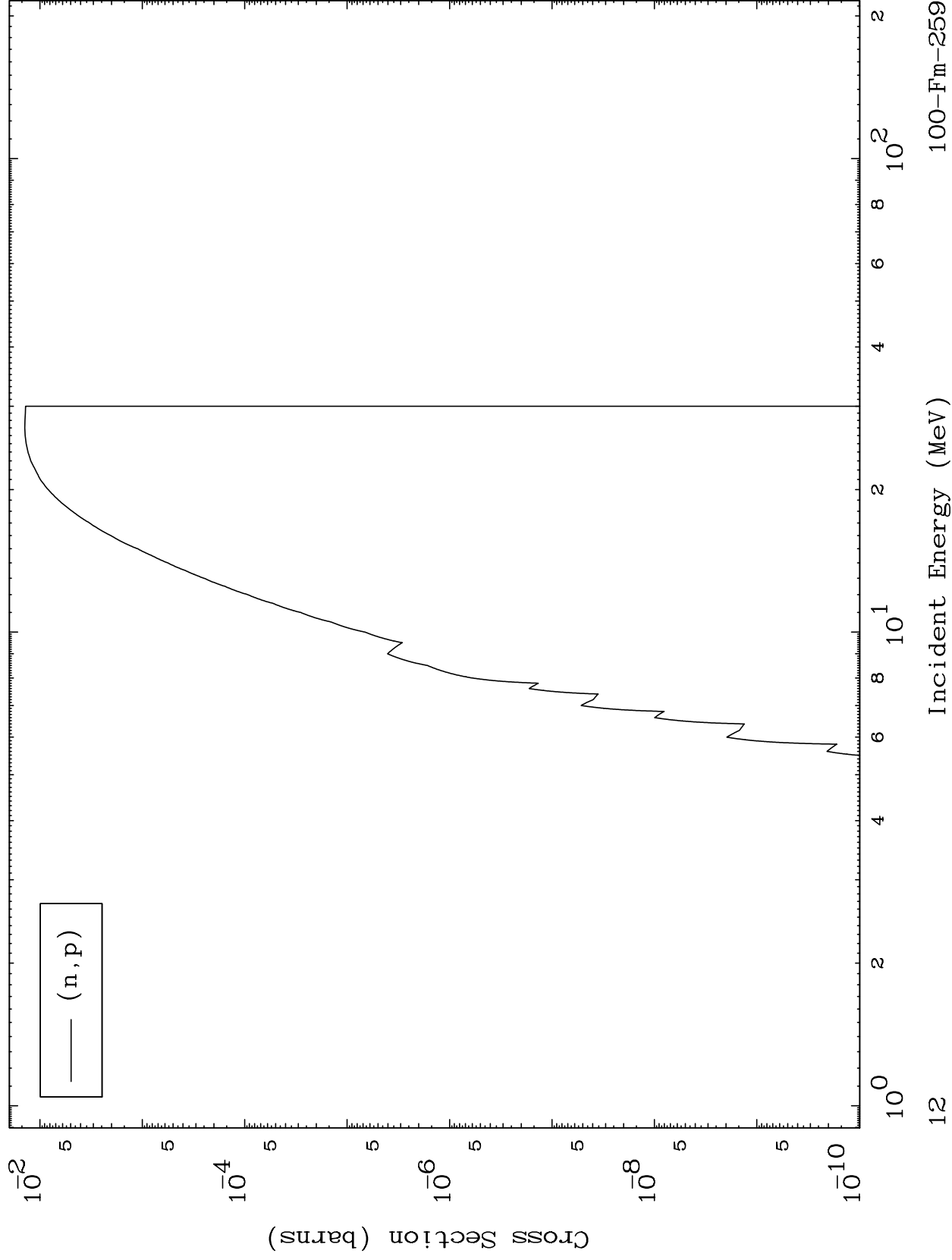
293 Kelvin Cross Sections

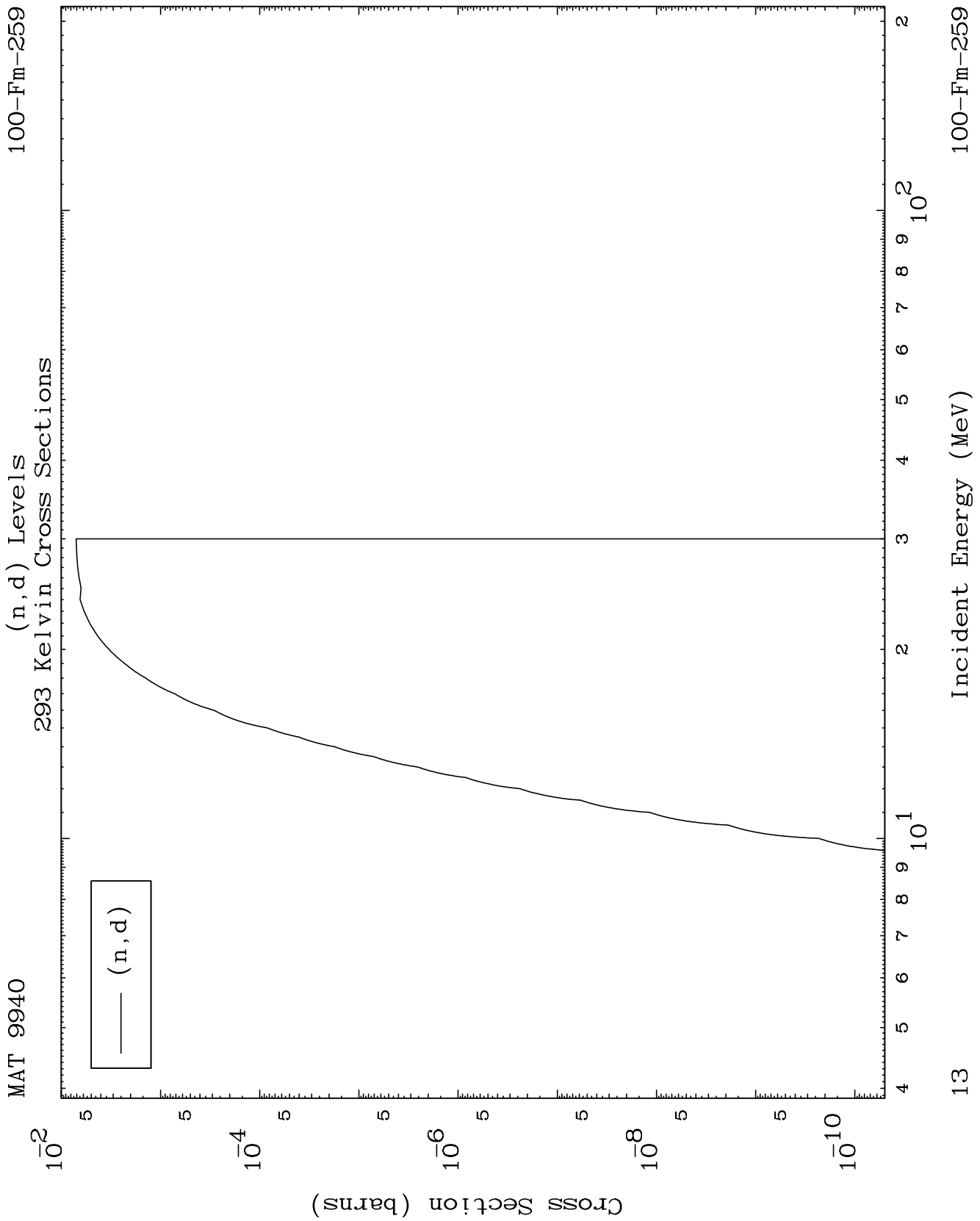


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(n,p) Levels
293 Kelvin Cross Sections

100-Fm-259

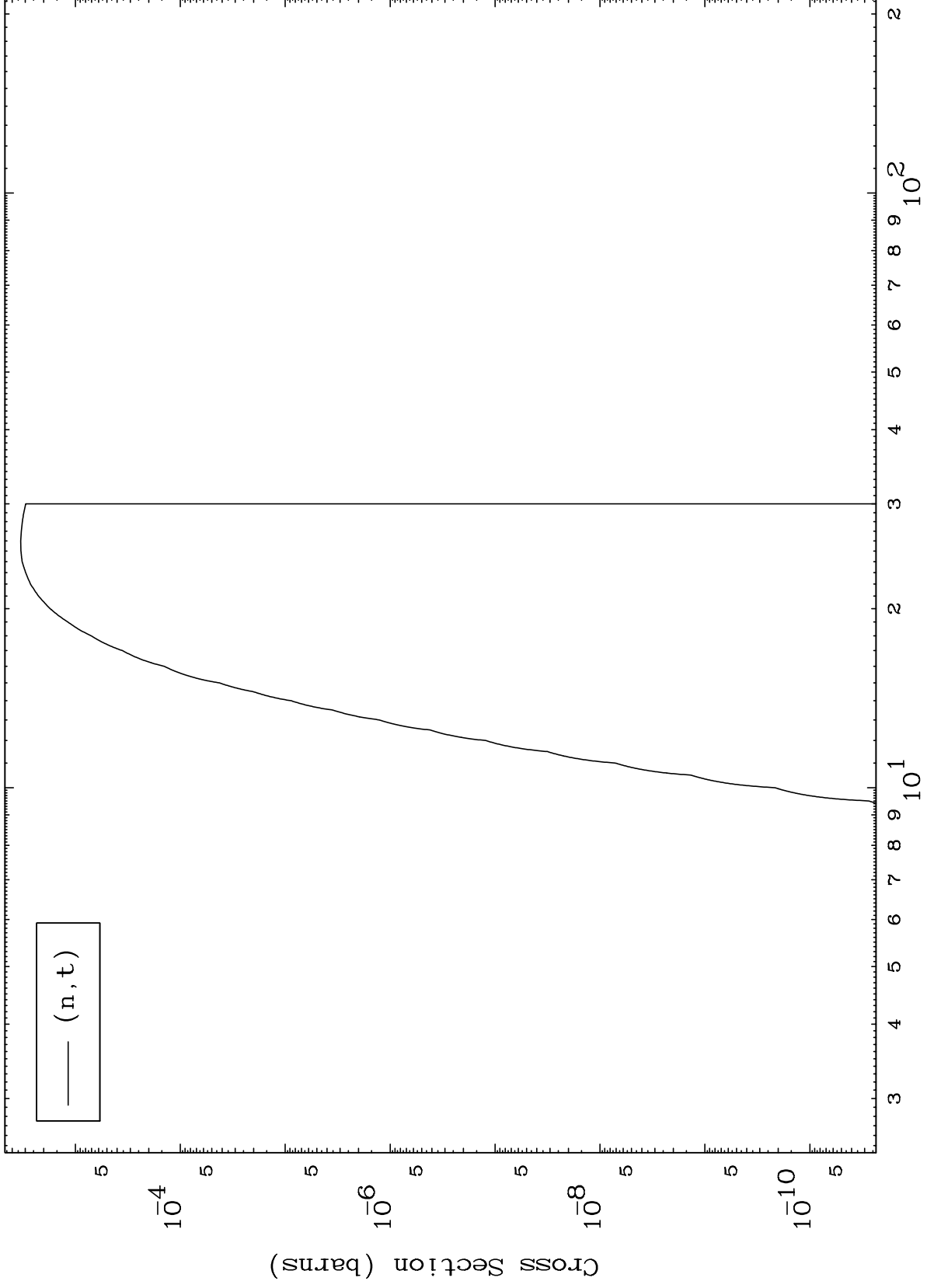




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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-259



14

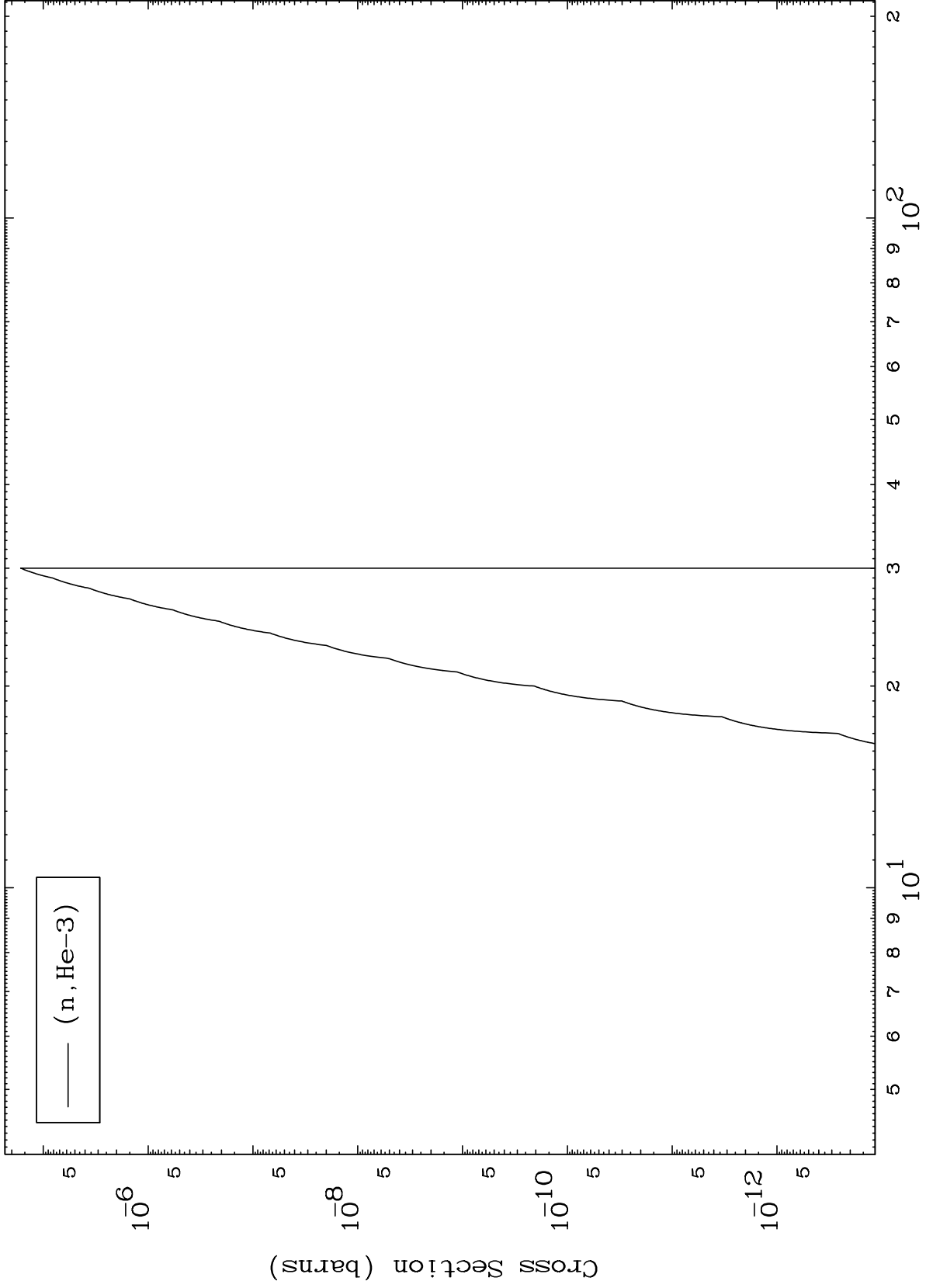
Incident Energy (MeV)

100-Fm-259

MAT 9940

(n,He3) Levels
293 Kelvin Cross Sections

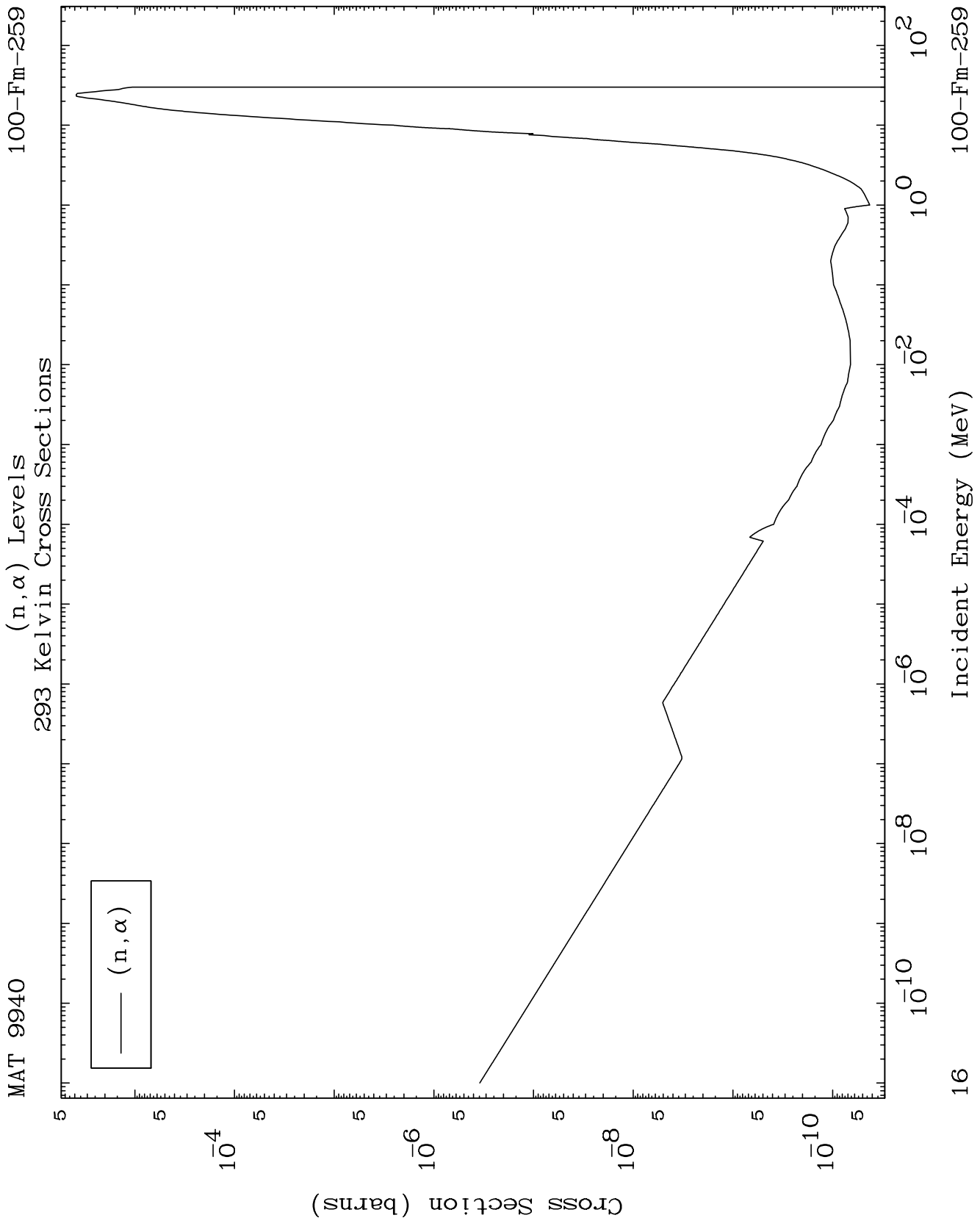
100-Fm-259



15

Incident Energy (MeV)

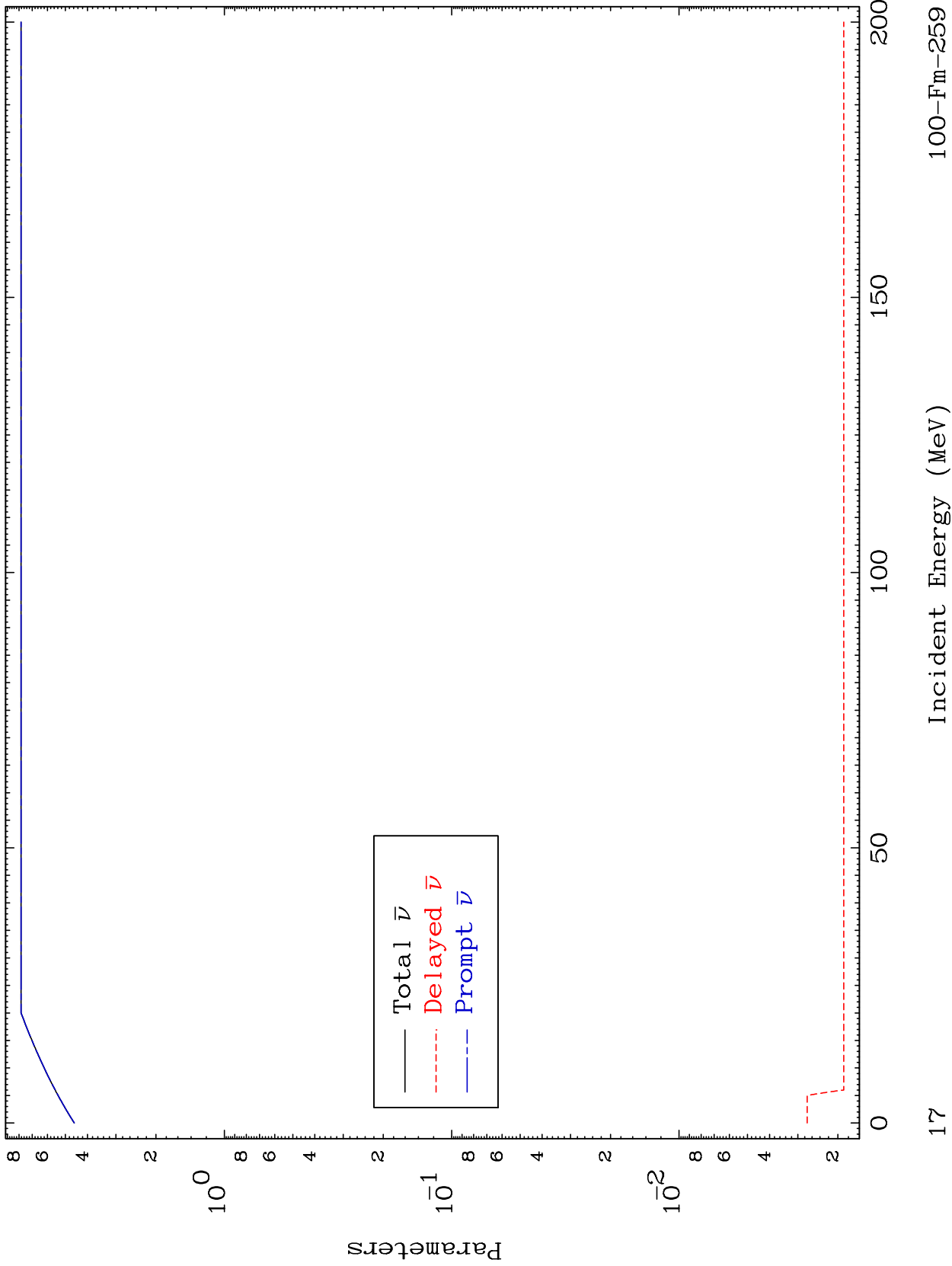
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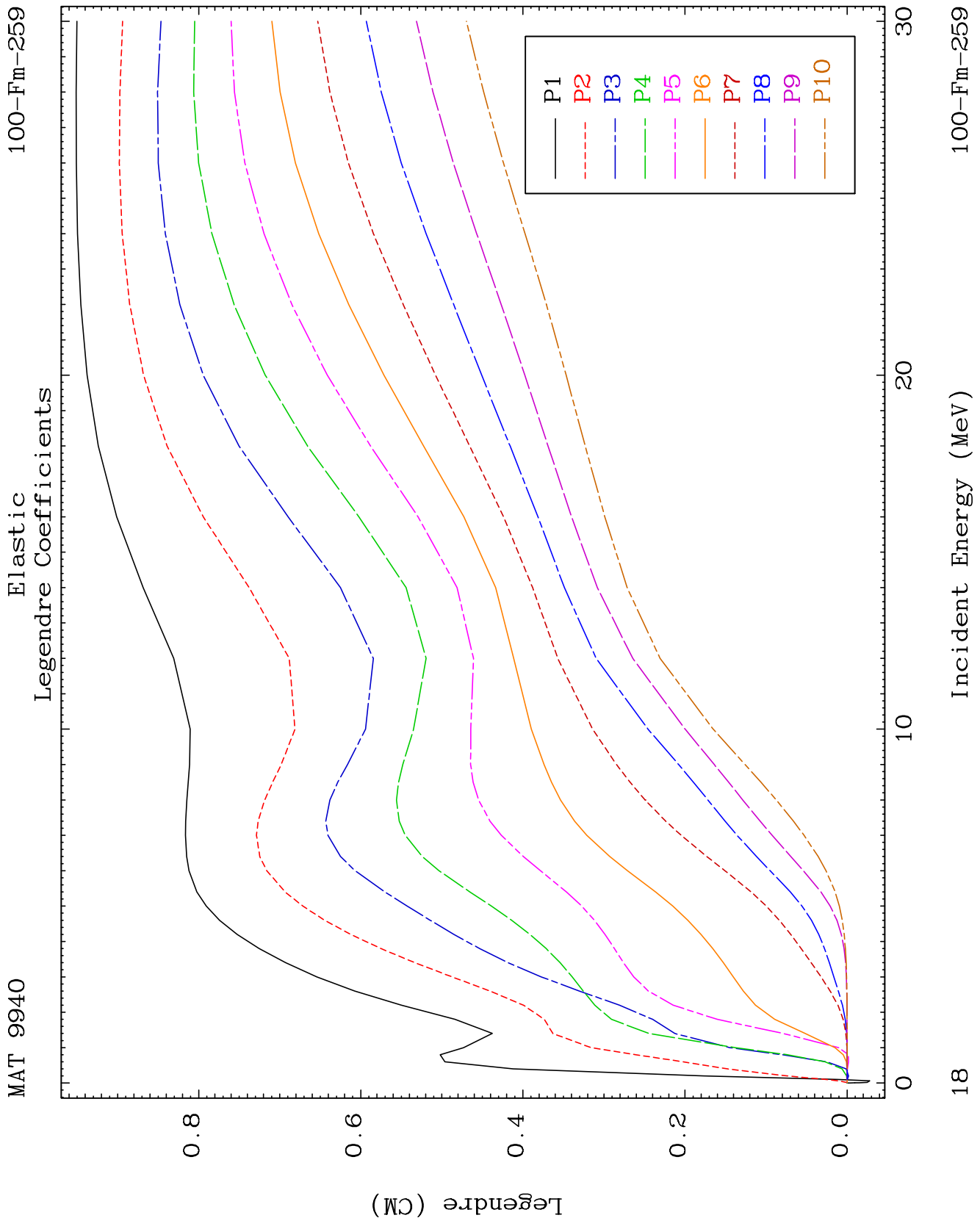


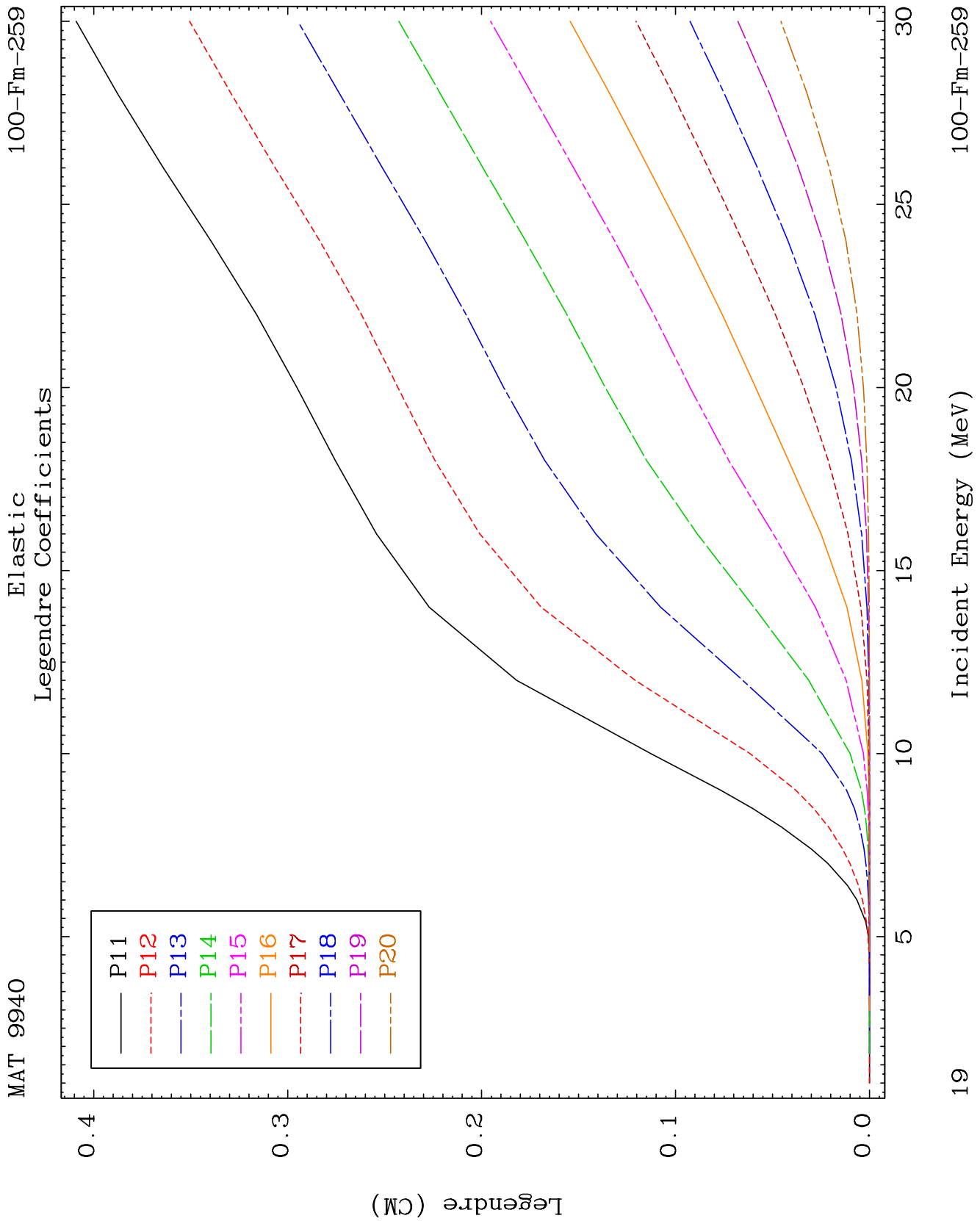
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Energy Release
Parameters

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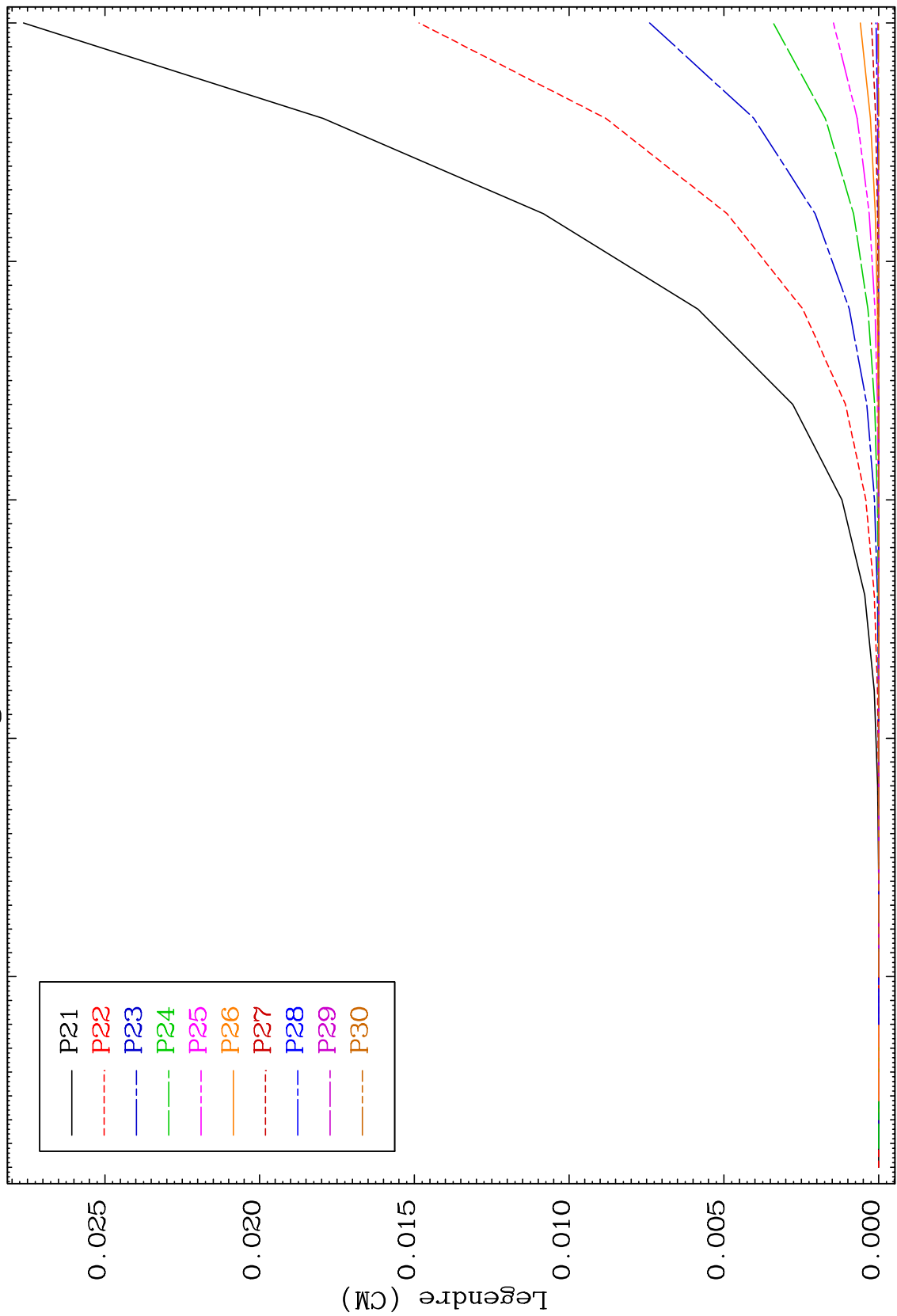




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Elastic Legendre Coefficients

100-Fm-259



100-Fm-259

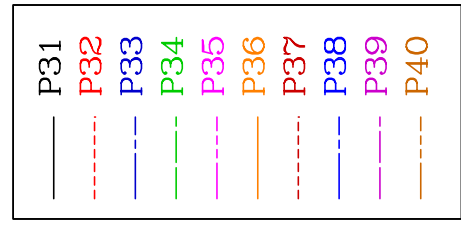
Incident Energy (MeV)

20

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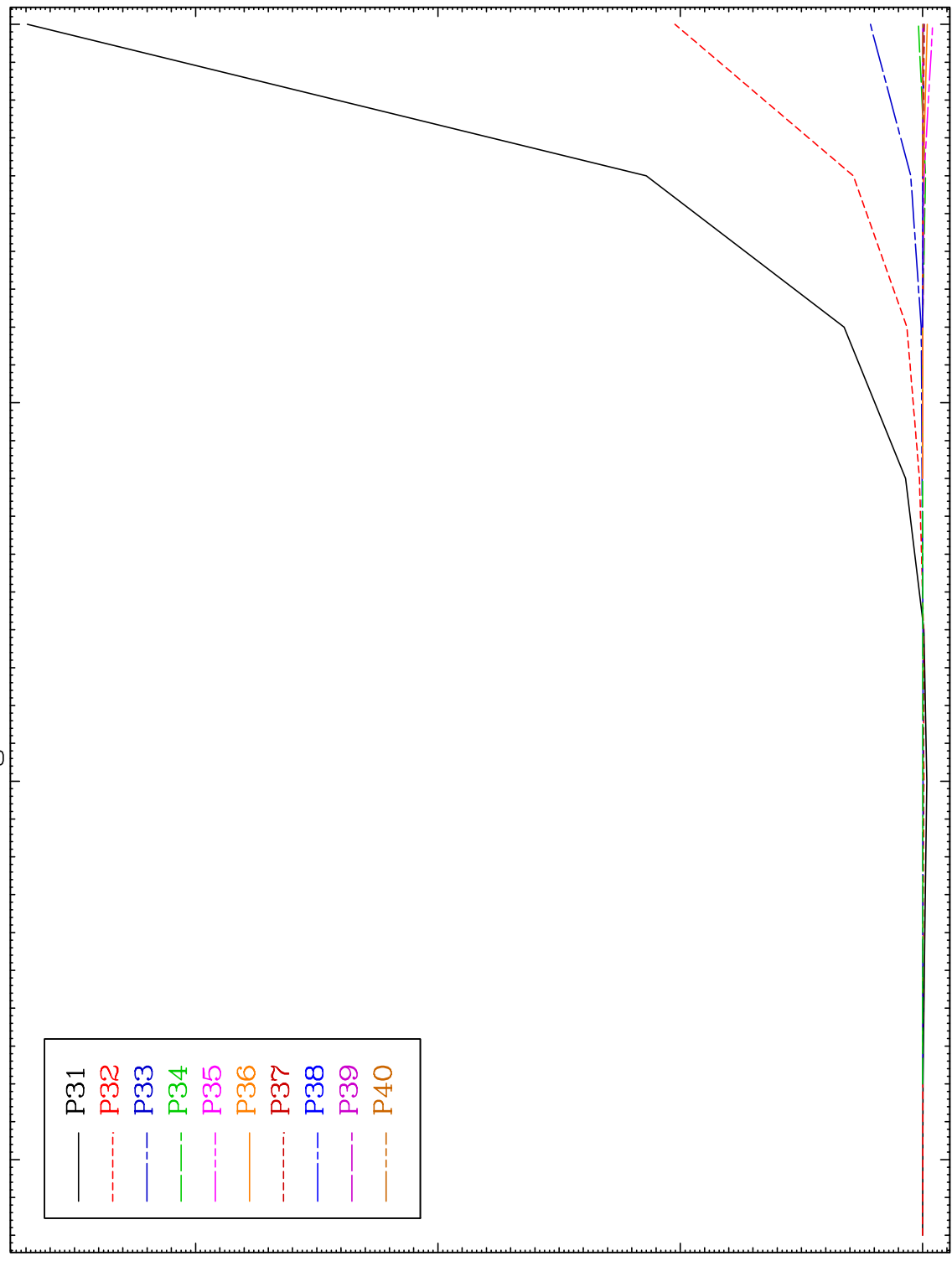
Elastic Legendre Coefficients

100-Fm-259



$\times 10^{-6}$

Legendre (CM)



21

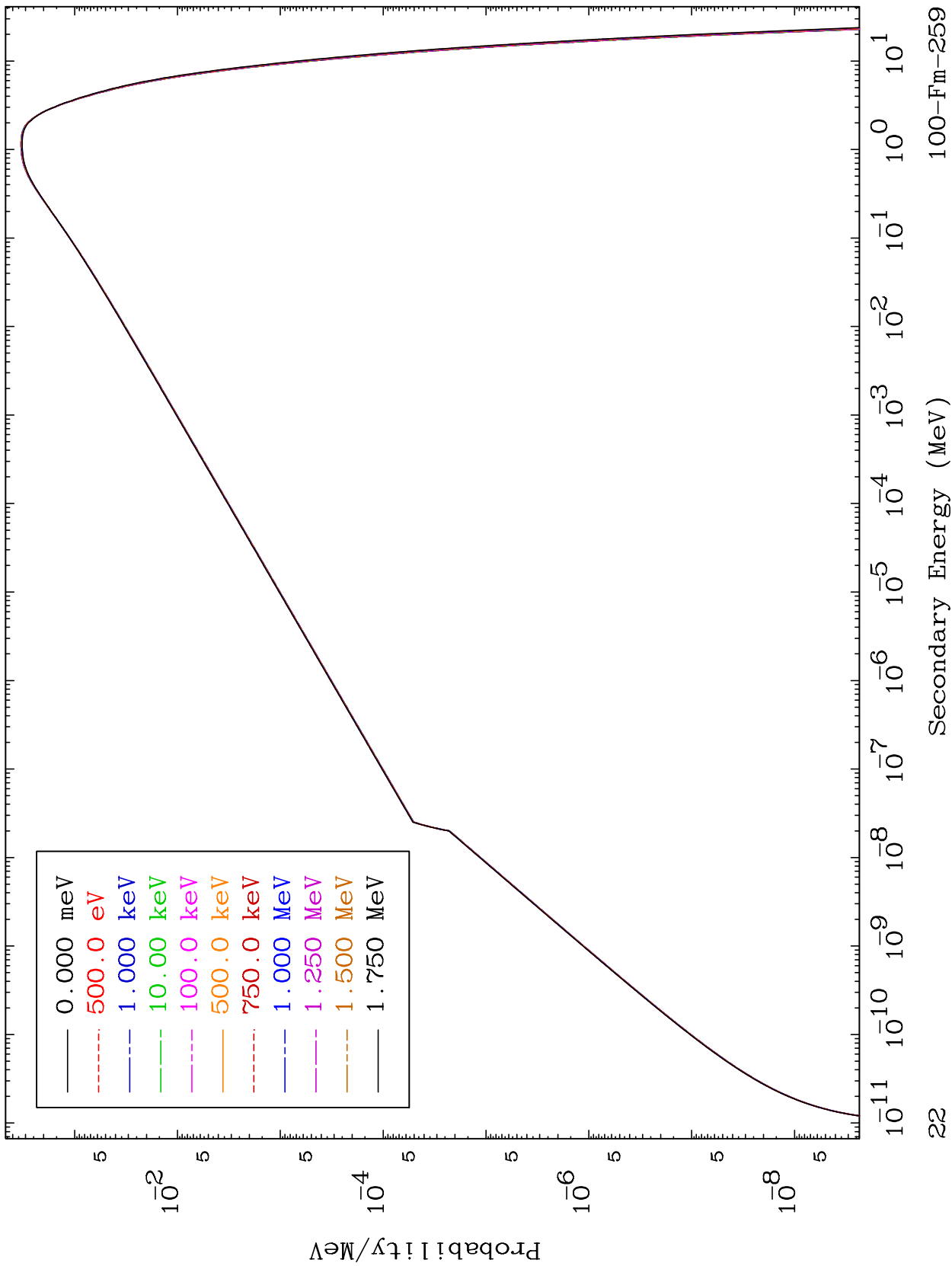
Incident Energy (MeV)

100-Fm-259

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Fission Energy Distributions

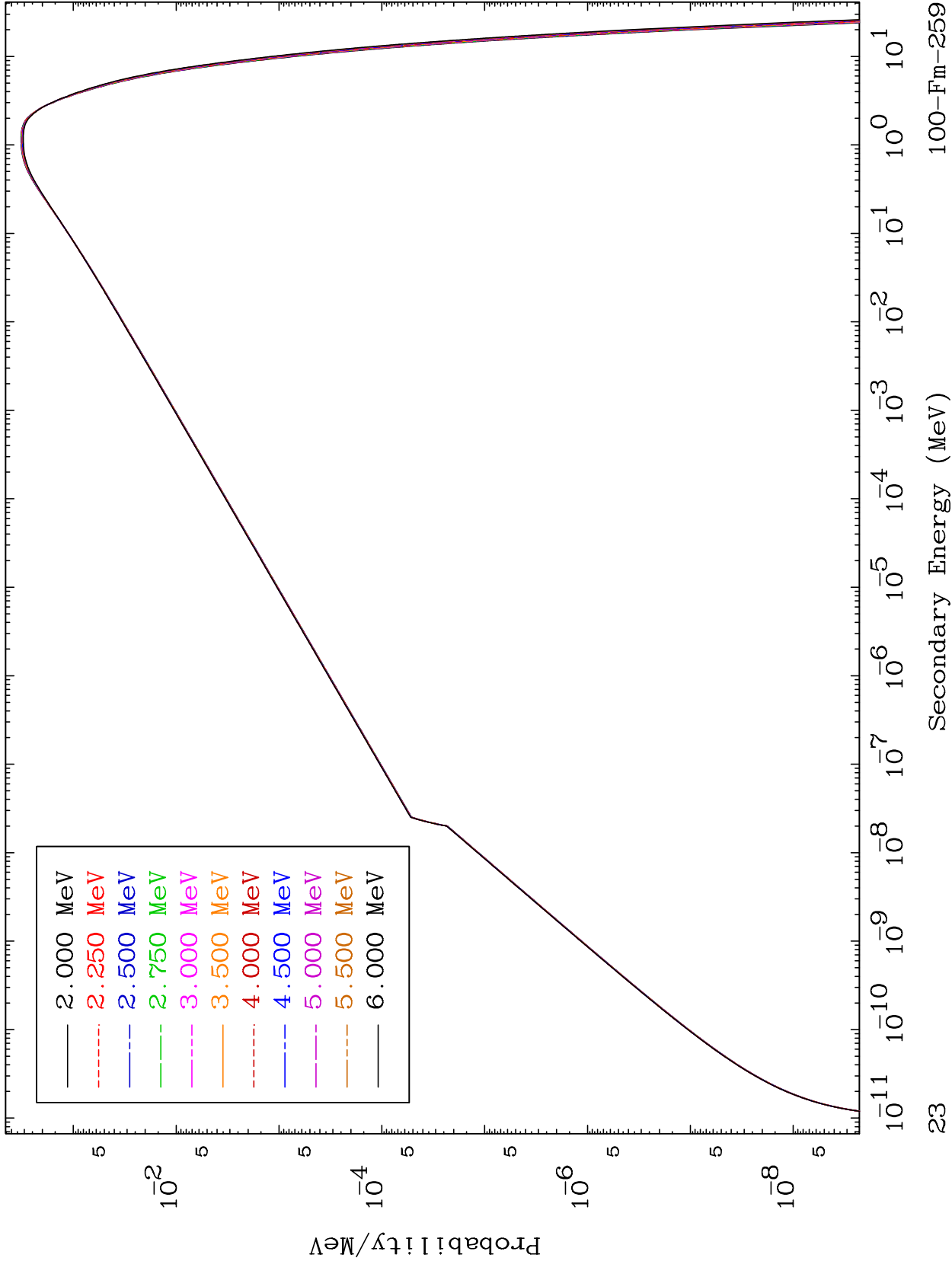
100-Fm-259



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Fission Energy Distributions

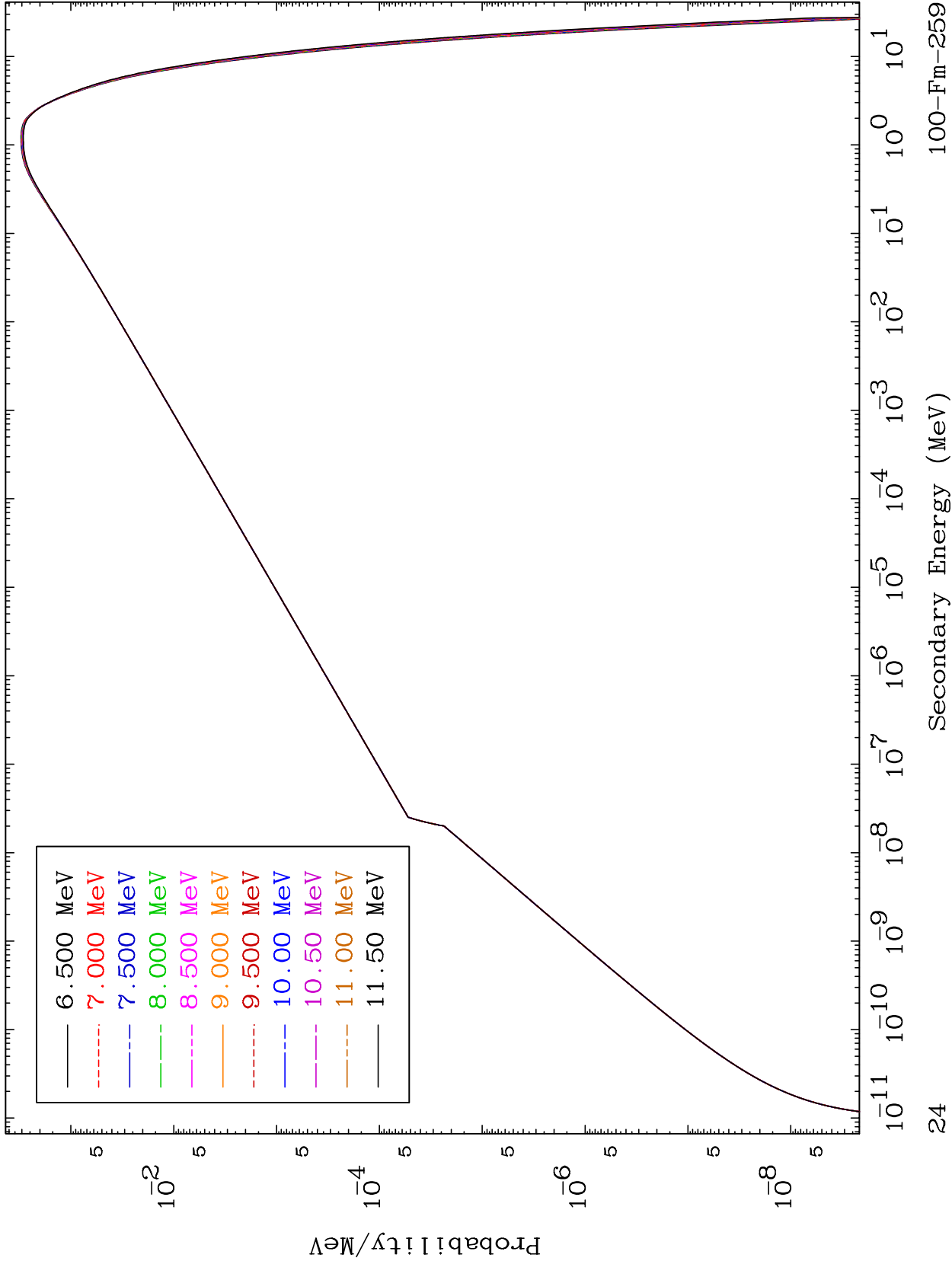
100-Fm-259



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Fission Energy Distributions

100-Fm-259



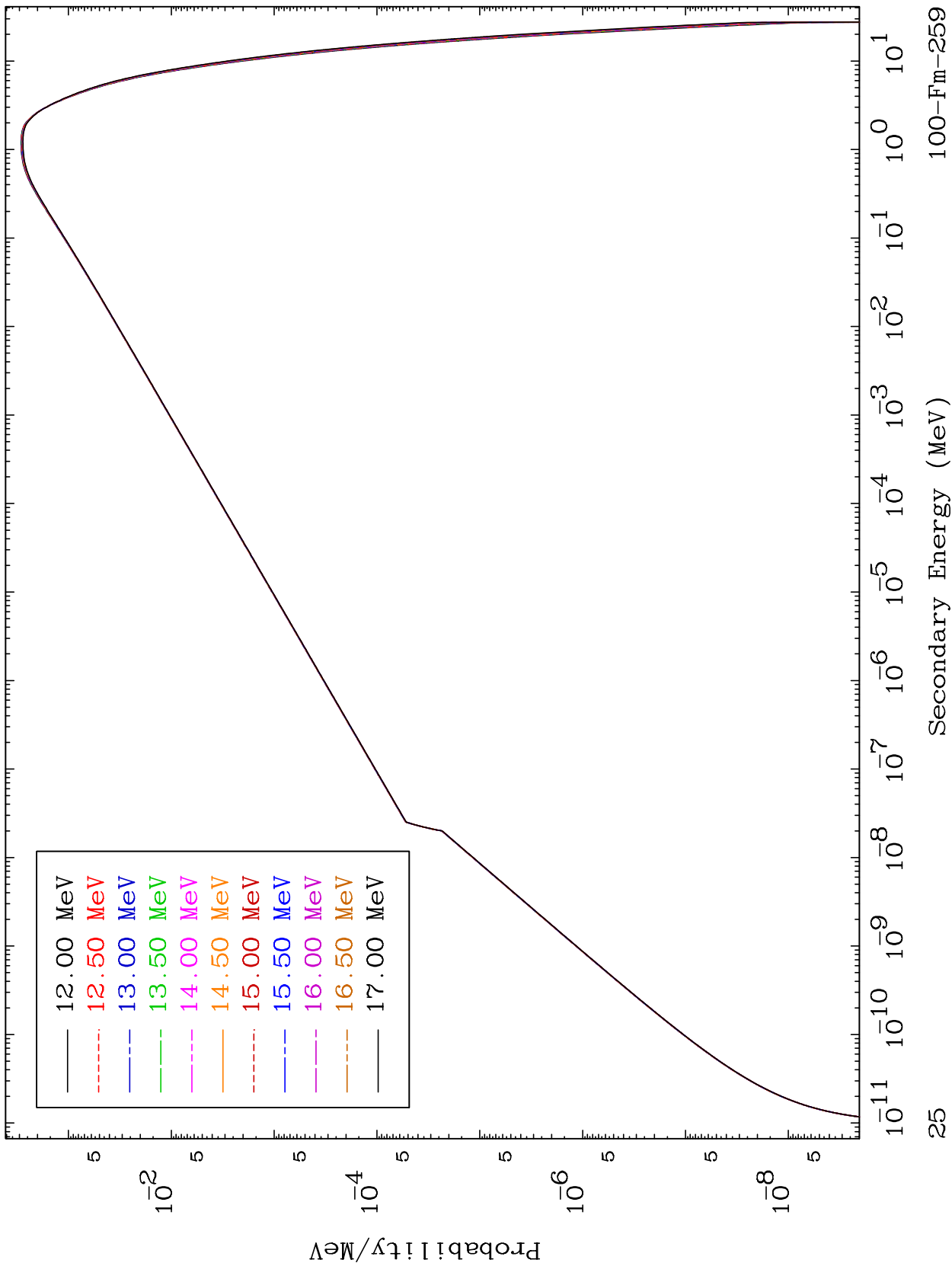
100-Fm-259

Secondary Energy (MeV)

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Fission Energy Distributions

100-Fm-259



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Fission Energy Distributions

100-Fm-259

