

Program EVALPLOT
(Version 2021-1)

by

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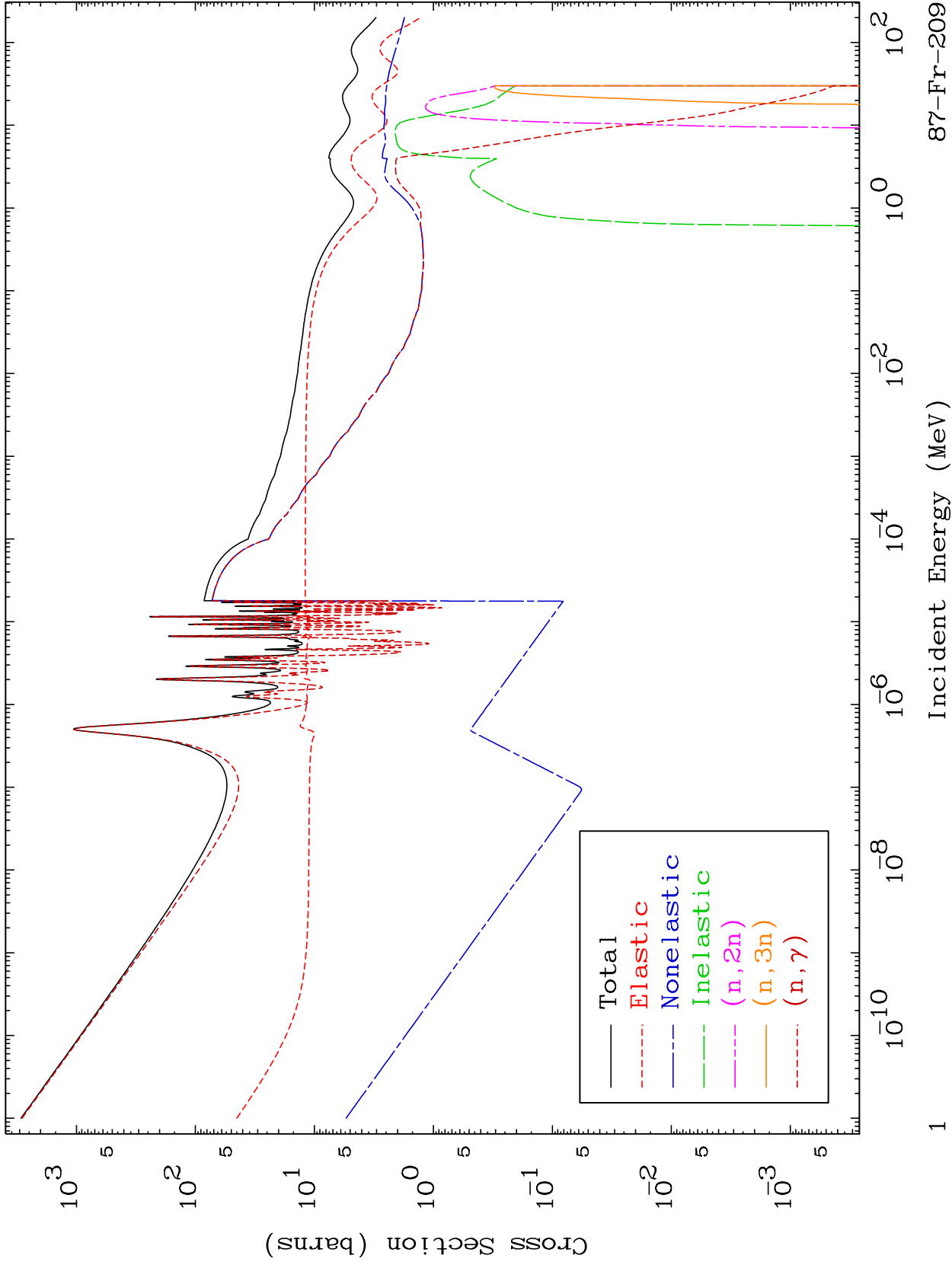
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Press Mouse Button to Start

MAT 8716

Neutron Major
293 Kelvin Cross Sections

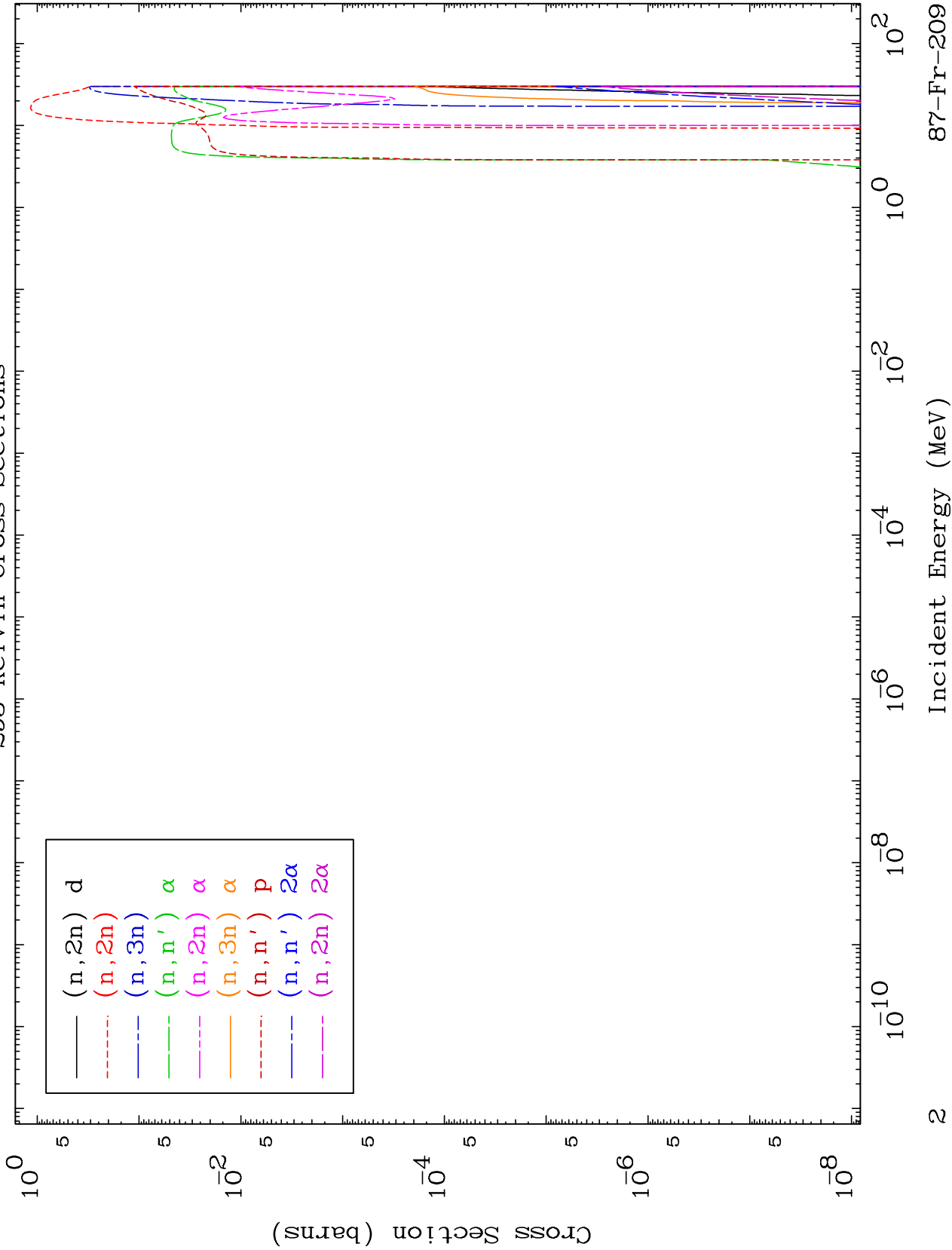
87-Fr-209



MAT 8716

Neutron Absorption
293 Kelvin Cross Sections

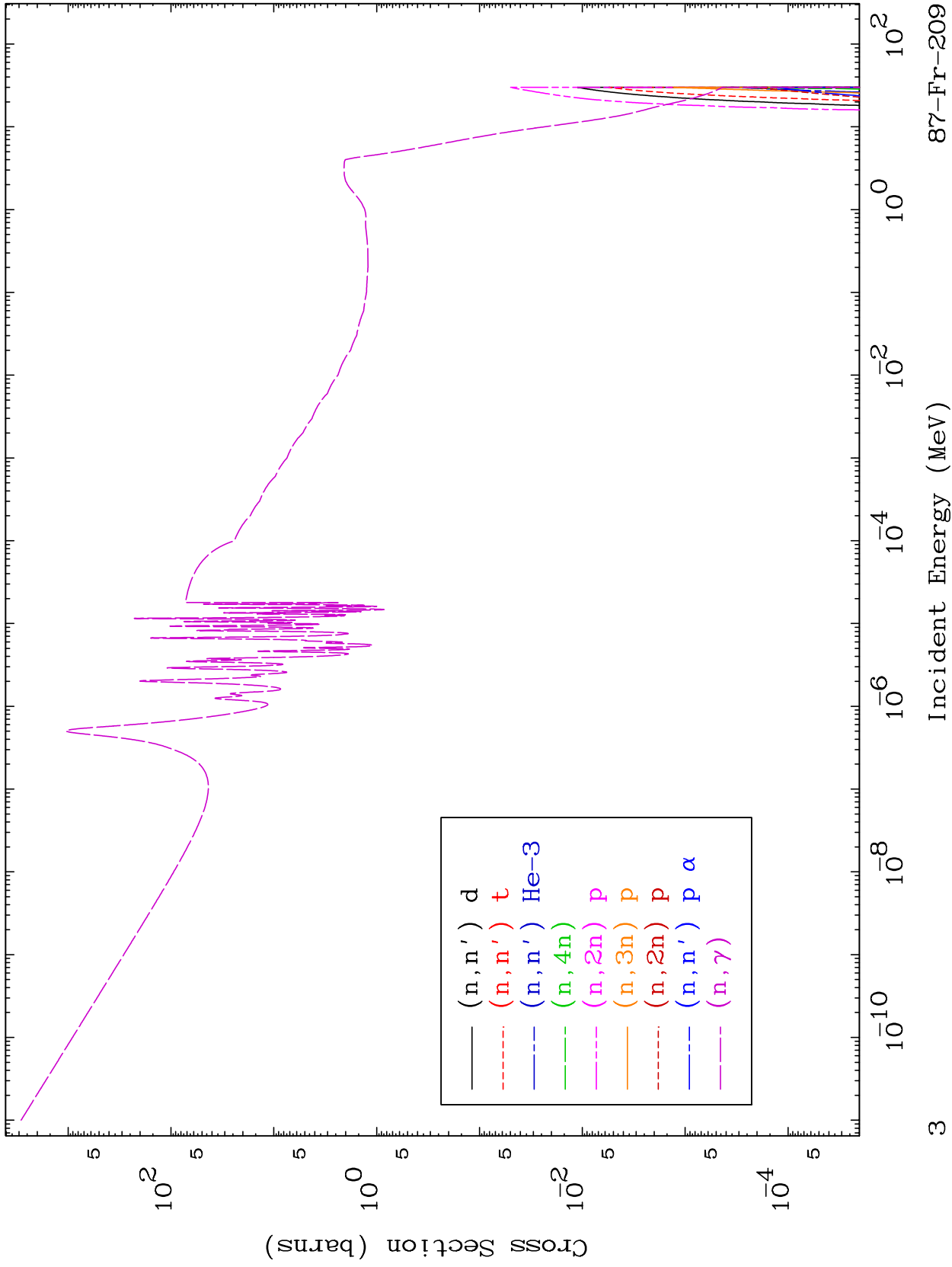
87-Fr-209



MAT 8716

Neutron Absorption
293 Kelvin Cross Sections

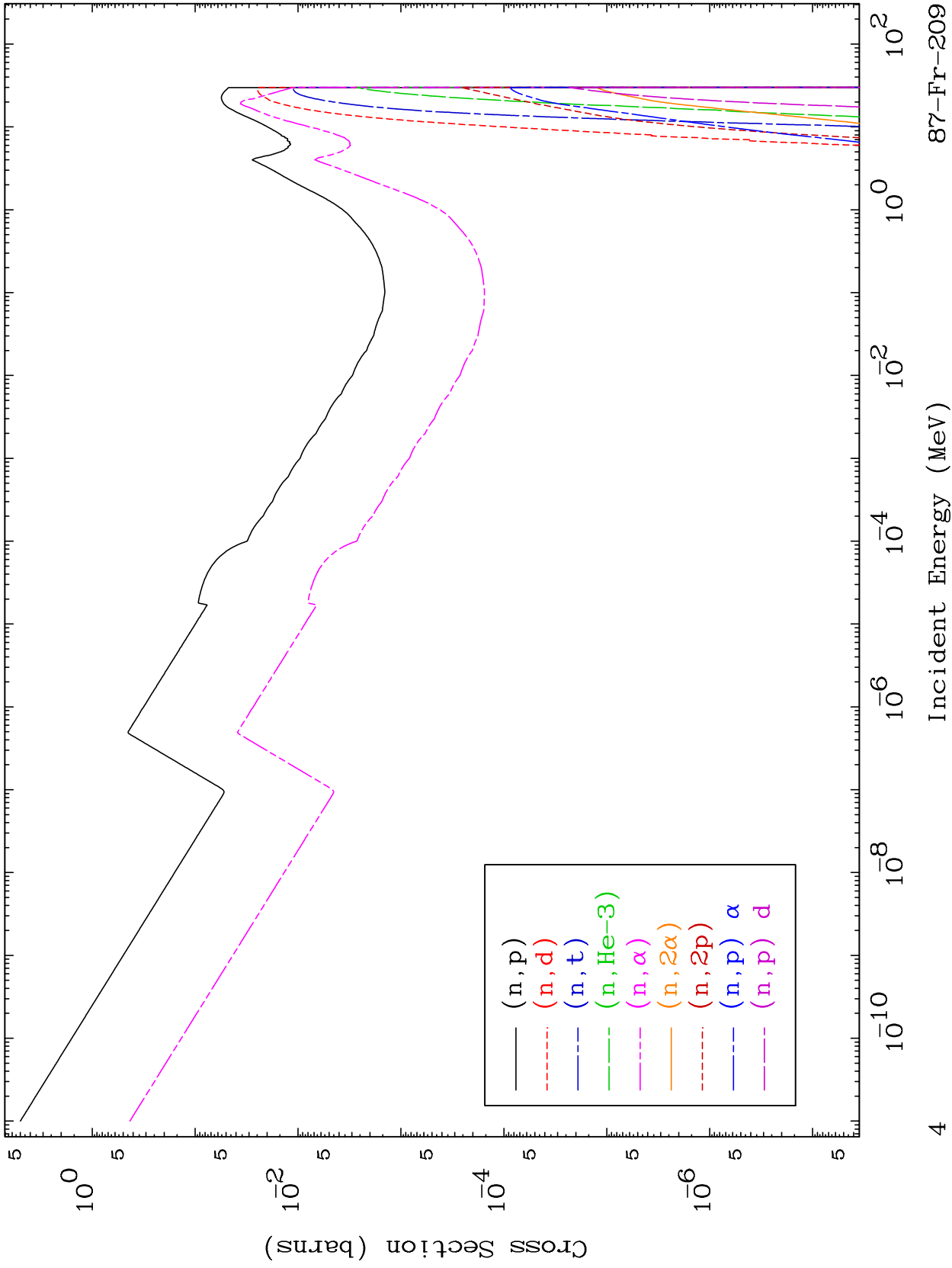
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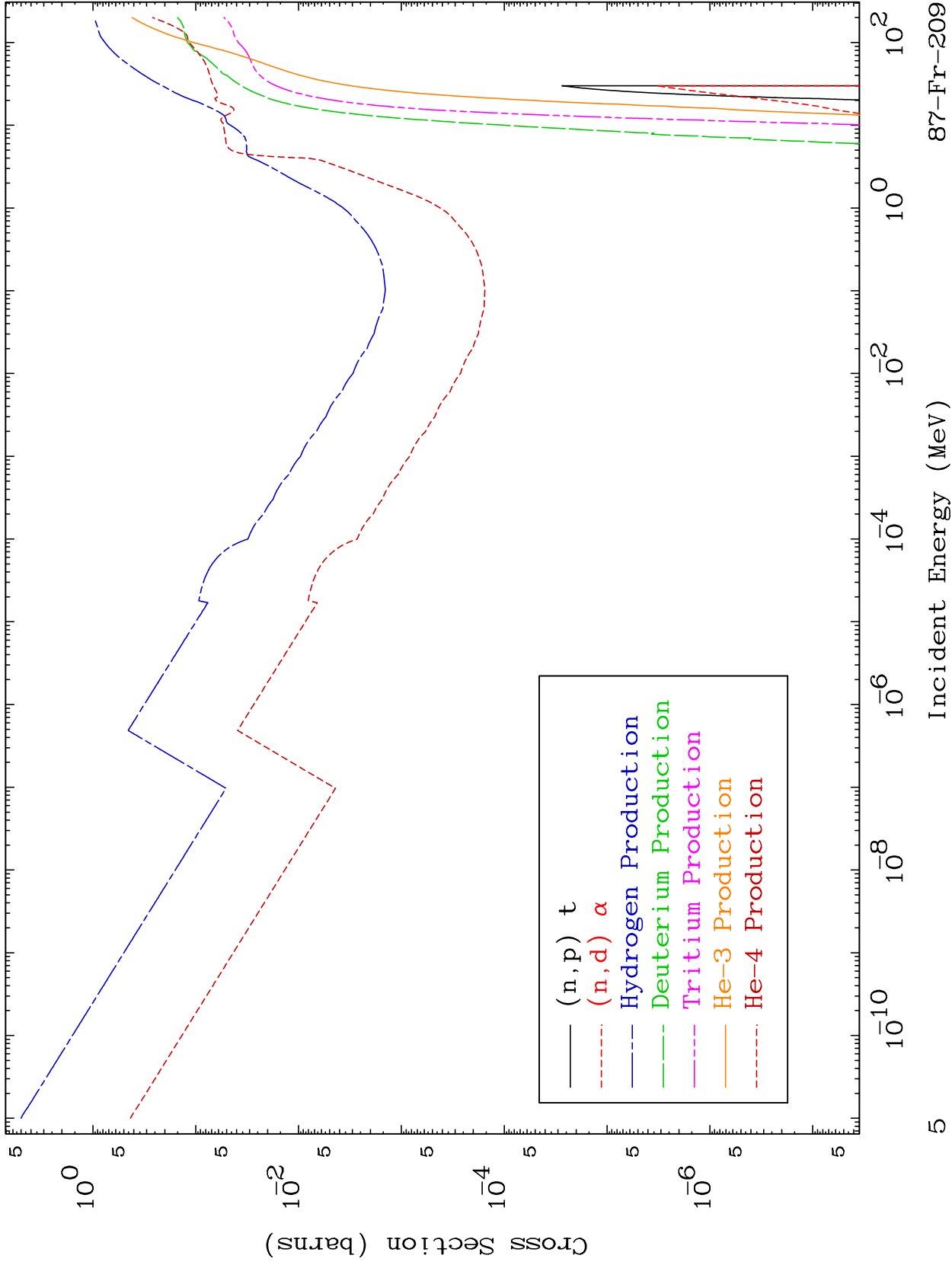


MAT 8716

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-209

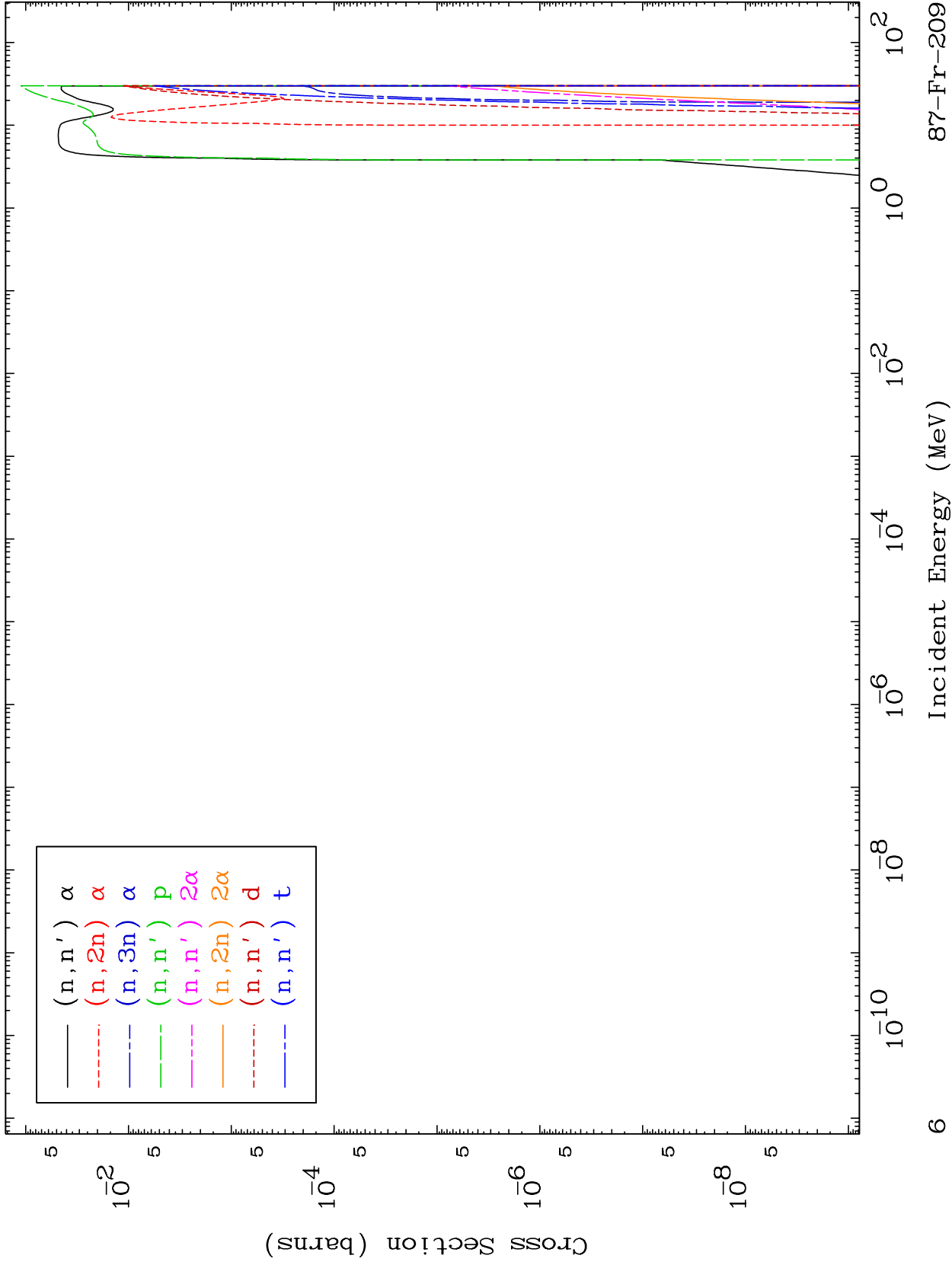




MAT 8716

Charged Particle
293 Kelvin Cross Sections

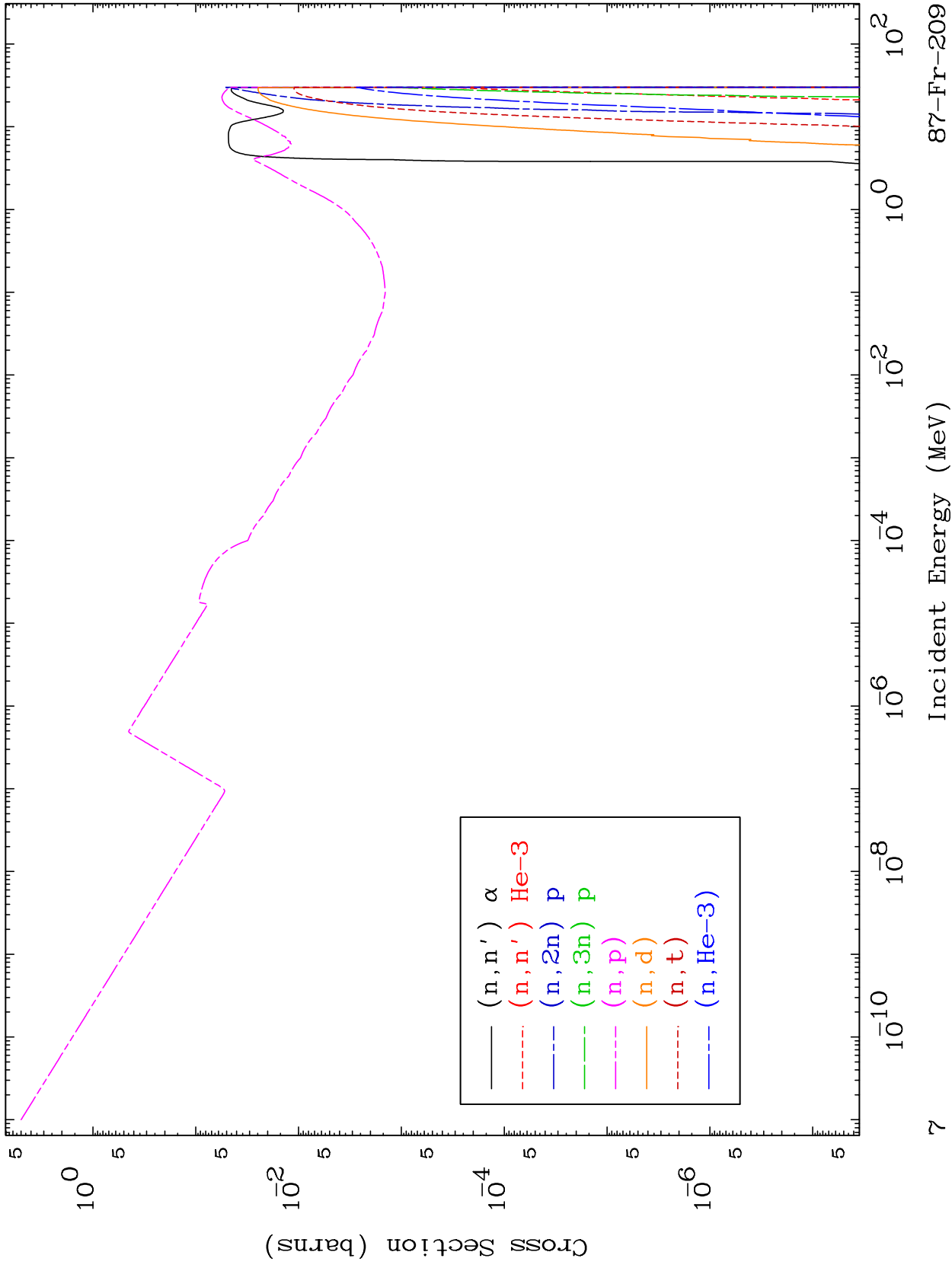
87-Fr-209



MAT 8716

Charged Particle
293 Kelvin Cross Sections

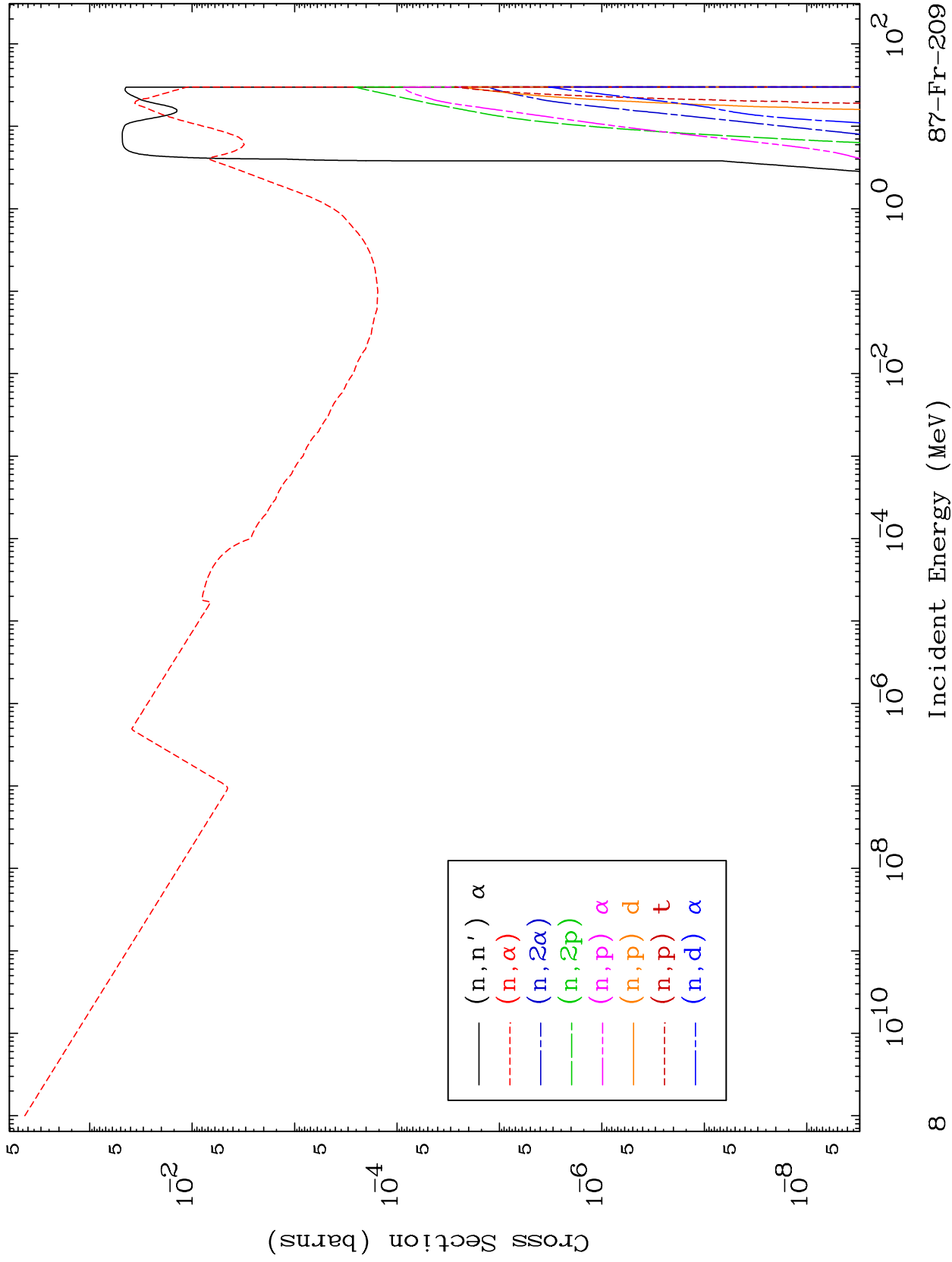
87-Fr-209



MAT 8716

Charged Particle
293 Kelvin Cross Sections

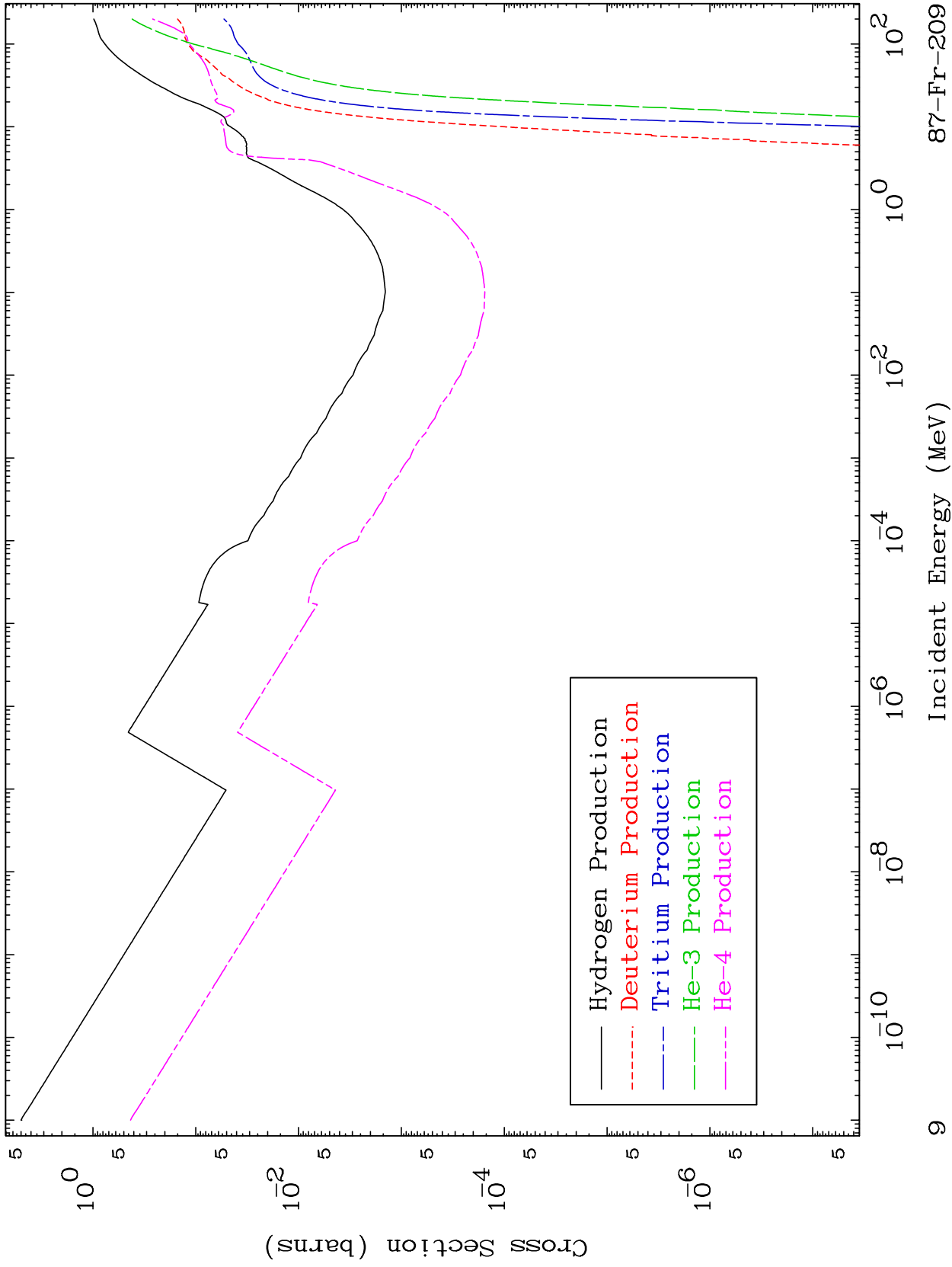
87-Fr-209



MAT 8716

Particle Production
293 Kelvin Cross Sections

87-Fr-209

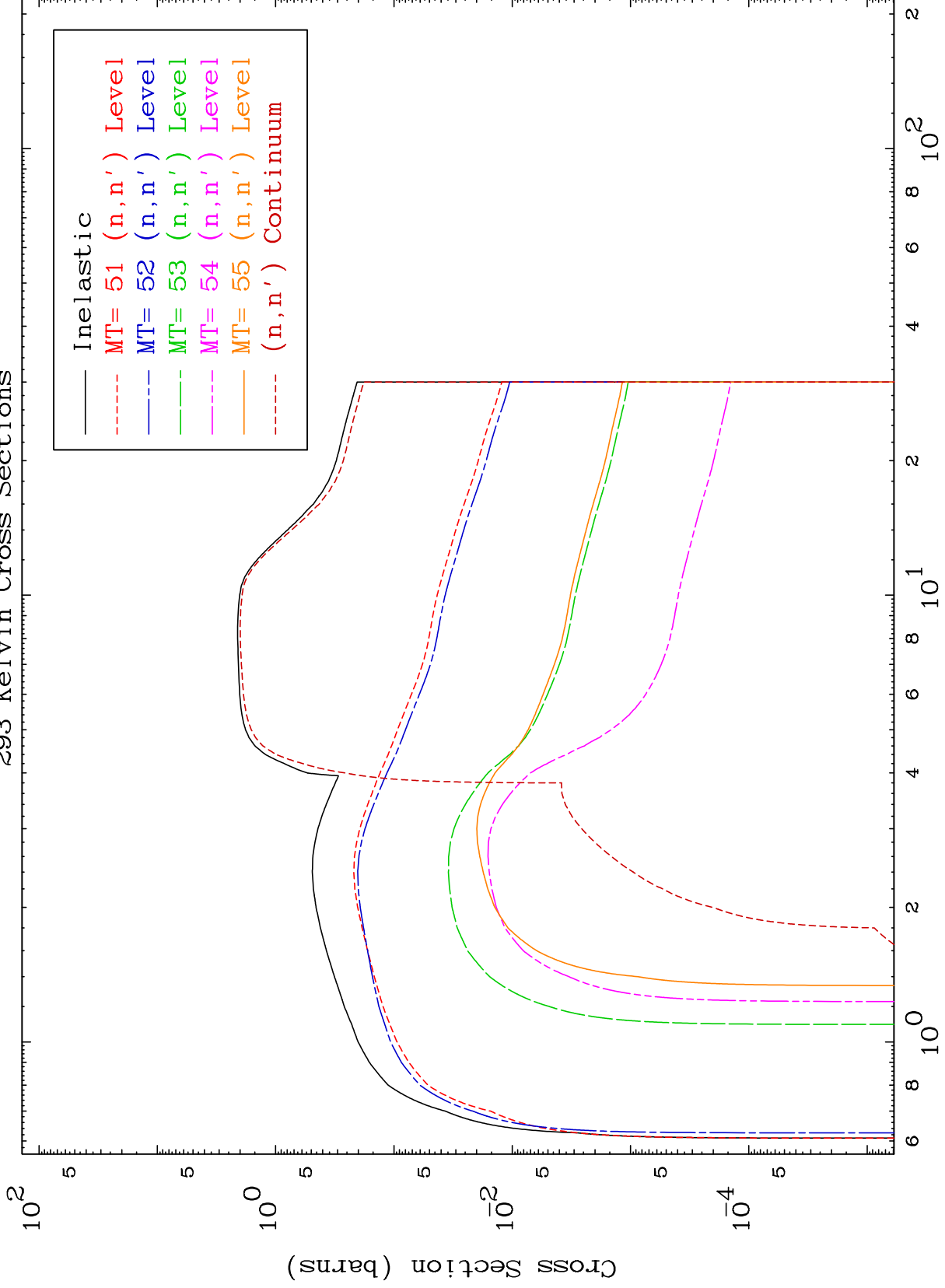


MAT 8716

(n,n') Levels

87-Fr-209

293 Kelvin Cross Sections



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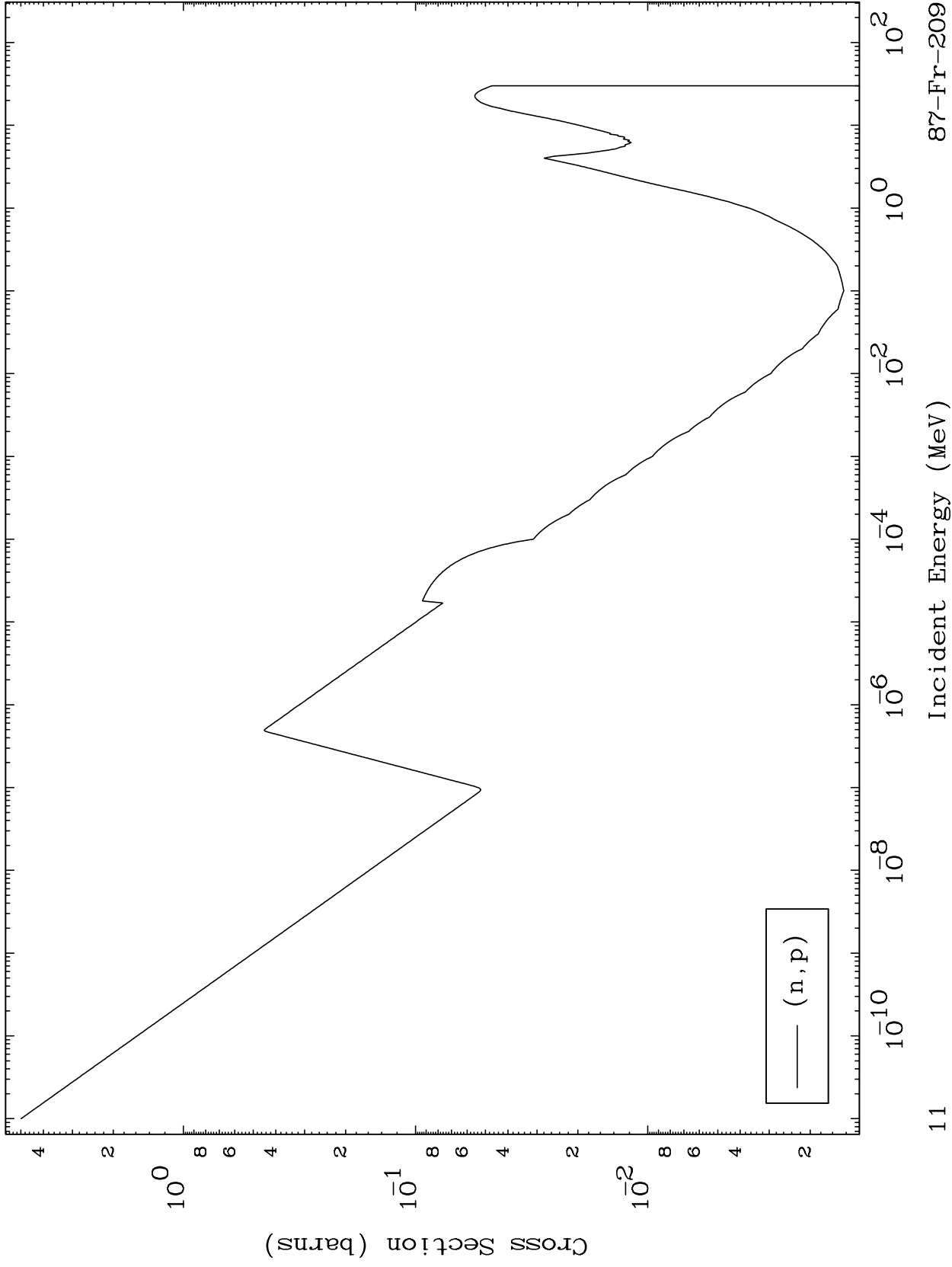
Incident Energy (MeV)

87-Fr-209

MAT 8716

(n,p) Levels
293 Kelvin Cross Sections

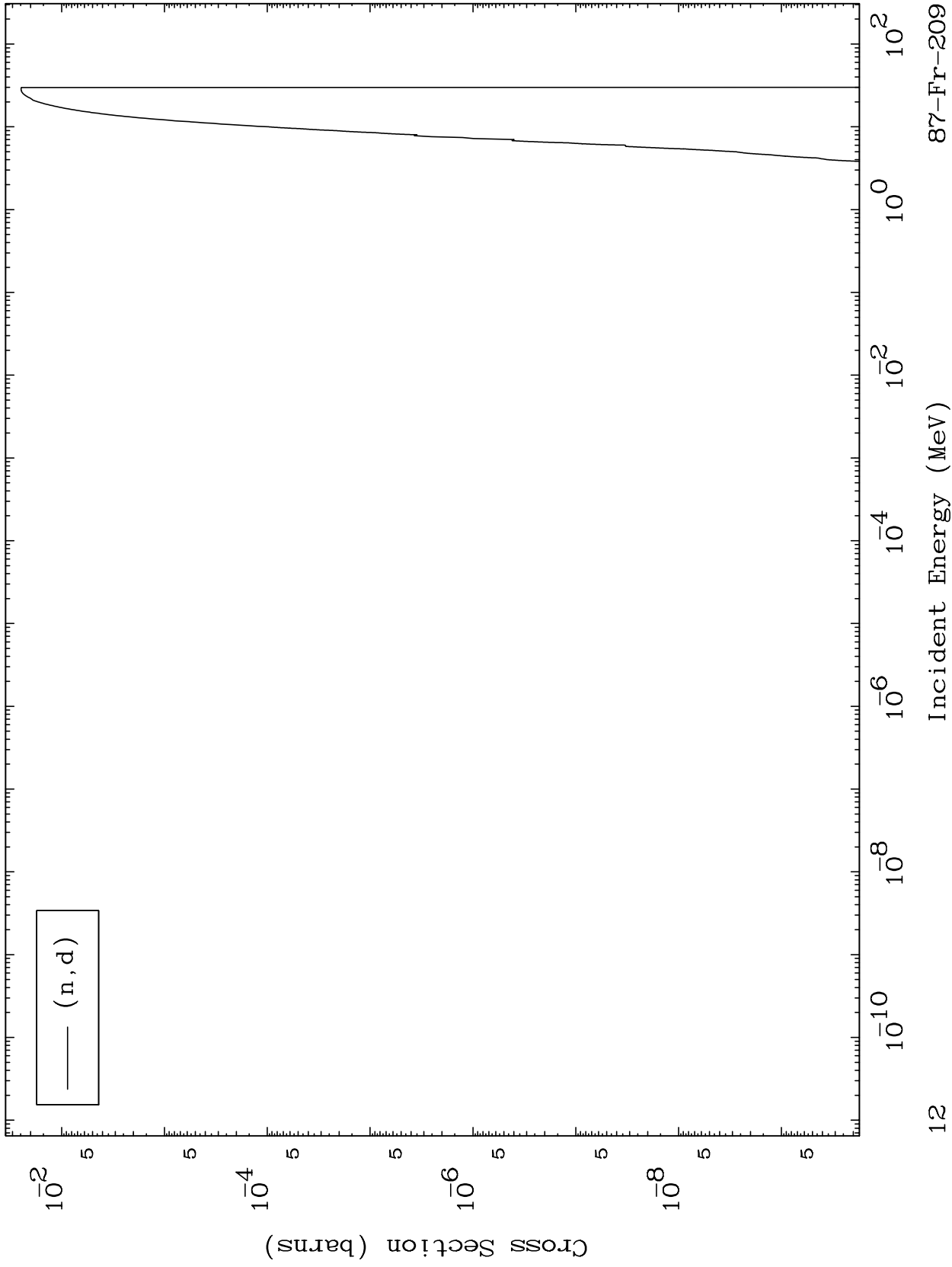
87-Fr-209



MAT 8716

(n,d) Levels
293 Kelvin Cross Sections

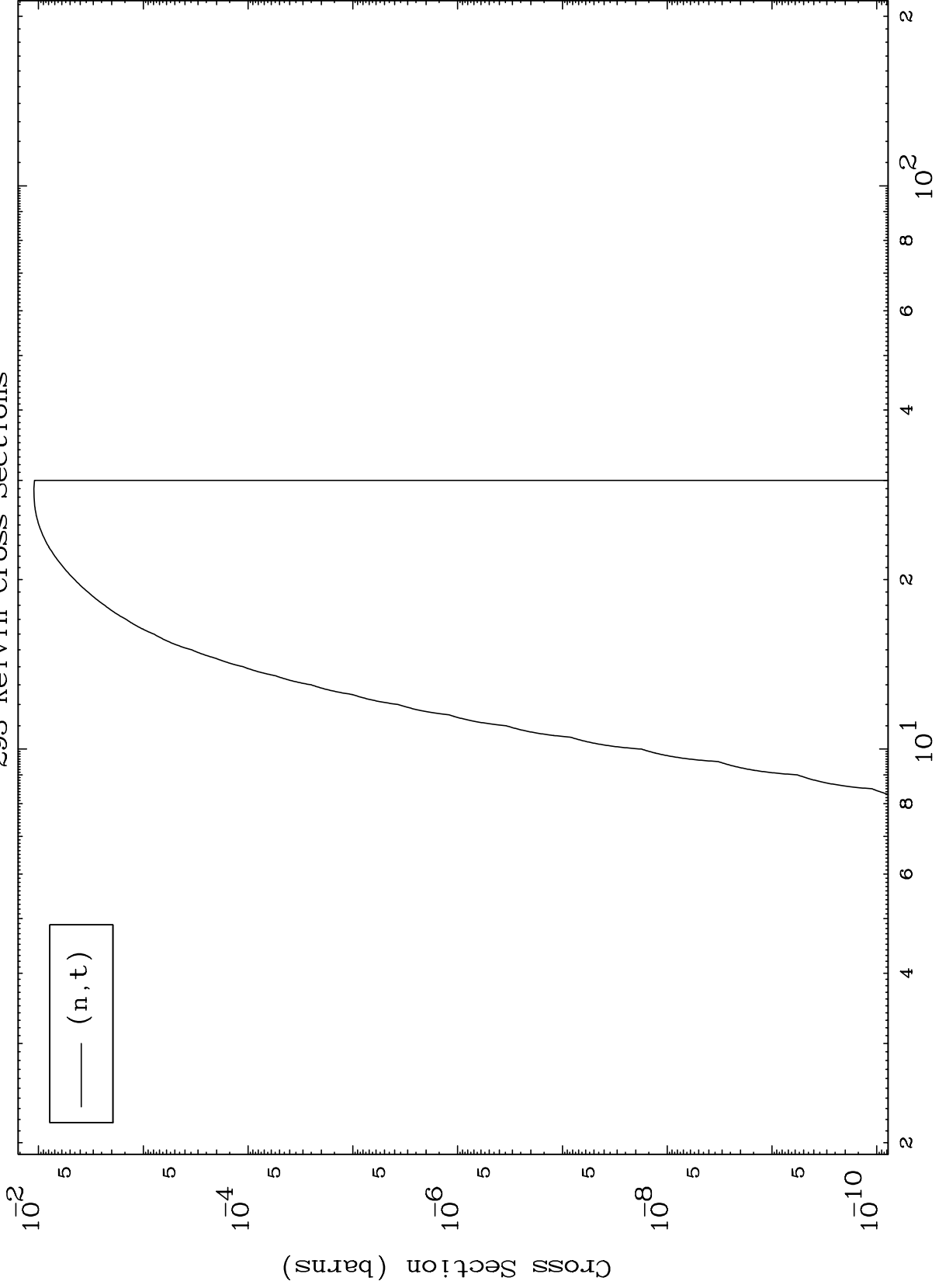
87-Fr-209



MAT 8716

(n,t) Levels
293 Kelvin Cross Sections

87-Fr-209



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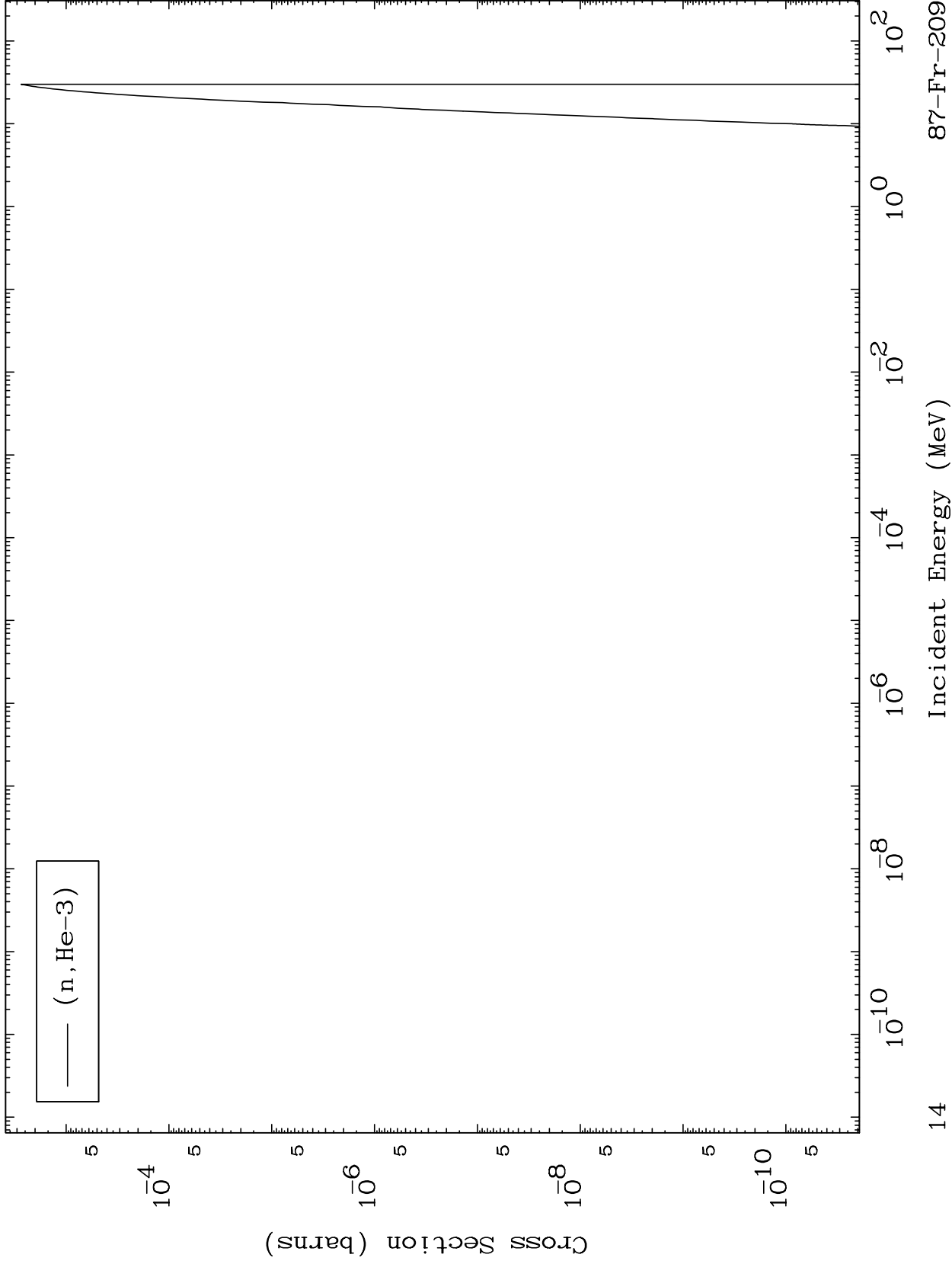
Incident Energy (MeV)

87-Fr-209

MAT 8716

(n,He3) Levels
293 Kelvin Cross Sections

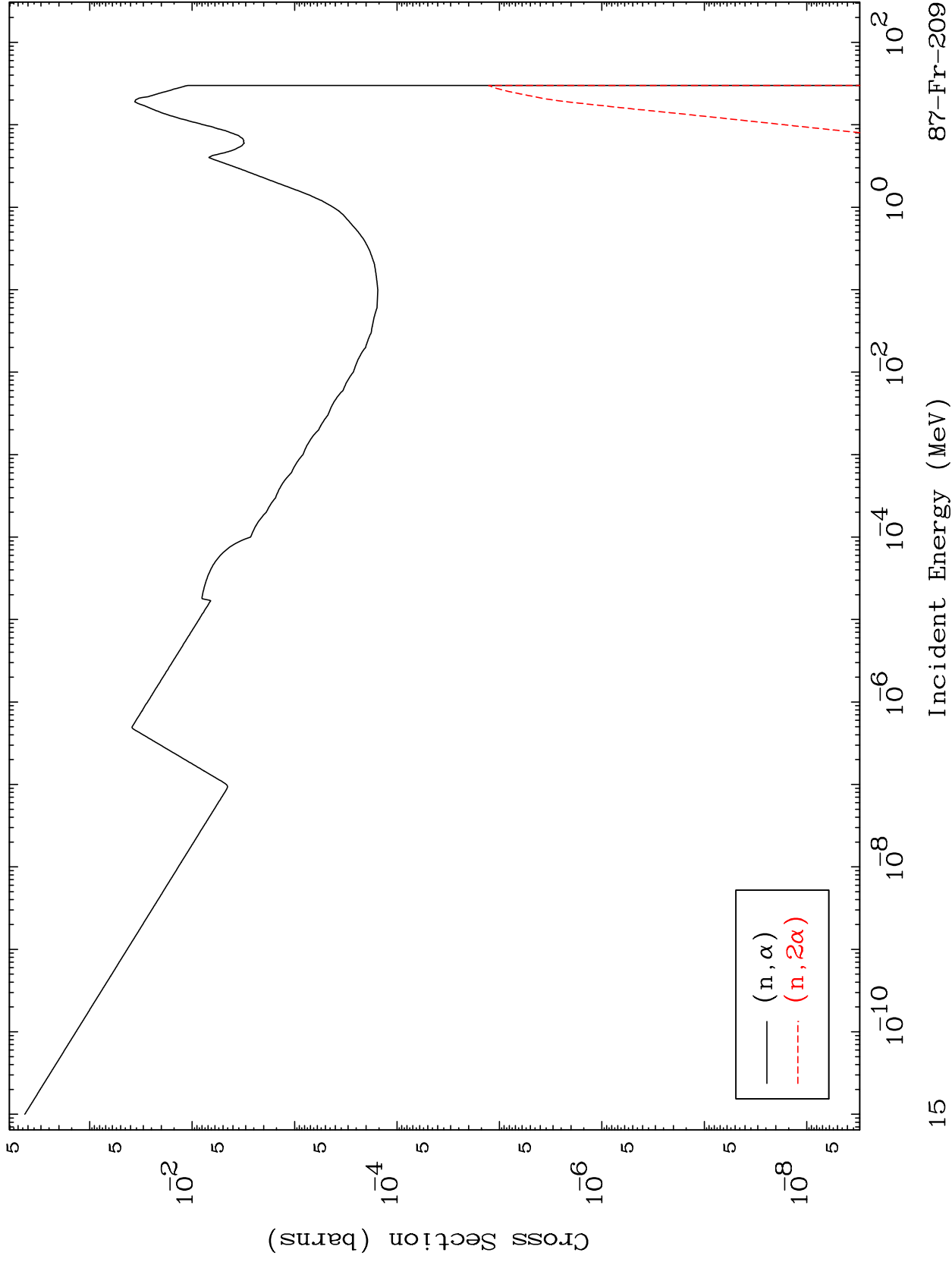
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MAT 8716

(n, α) Levels
293 Kelvin Cross Sections

87-Fr-209



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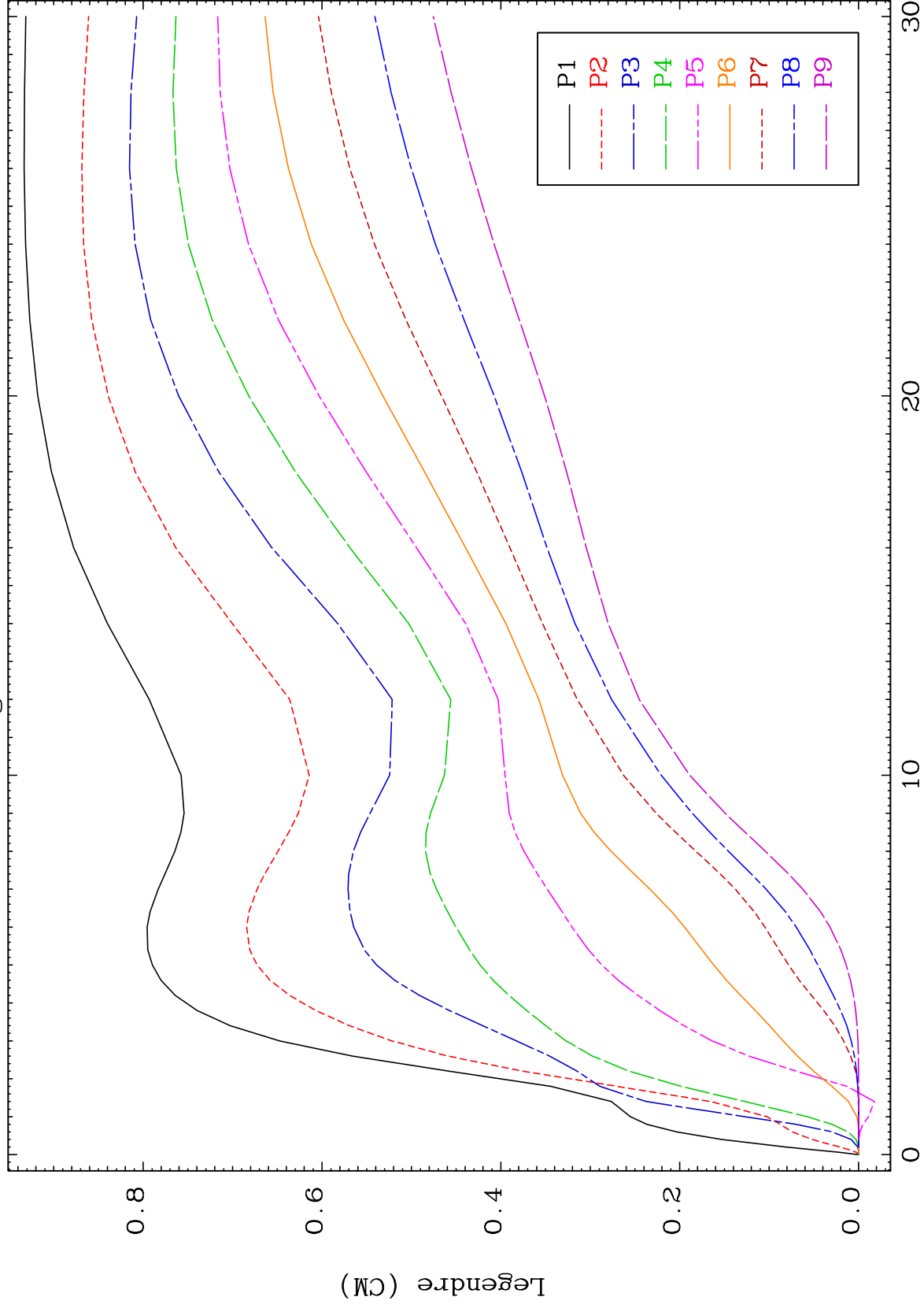
Incident Energy (MeV)

87-Fr-209

MAT 8716

Elastic Legendre Coefficients

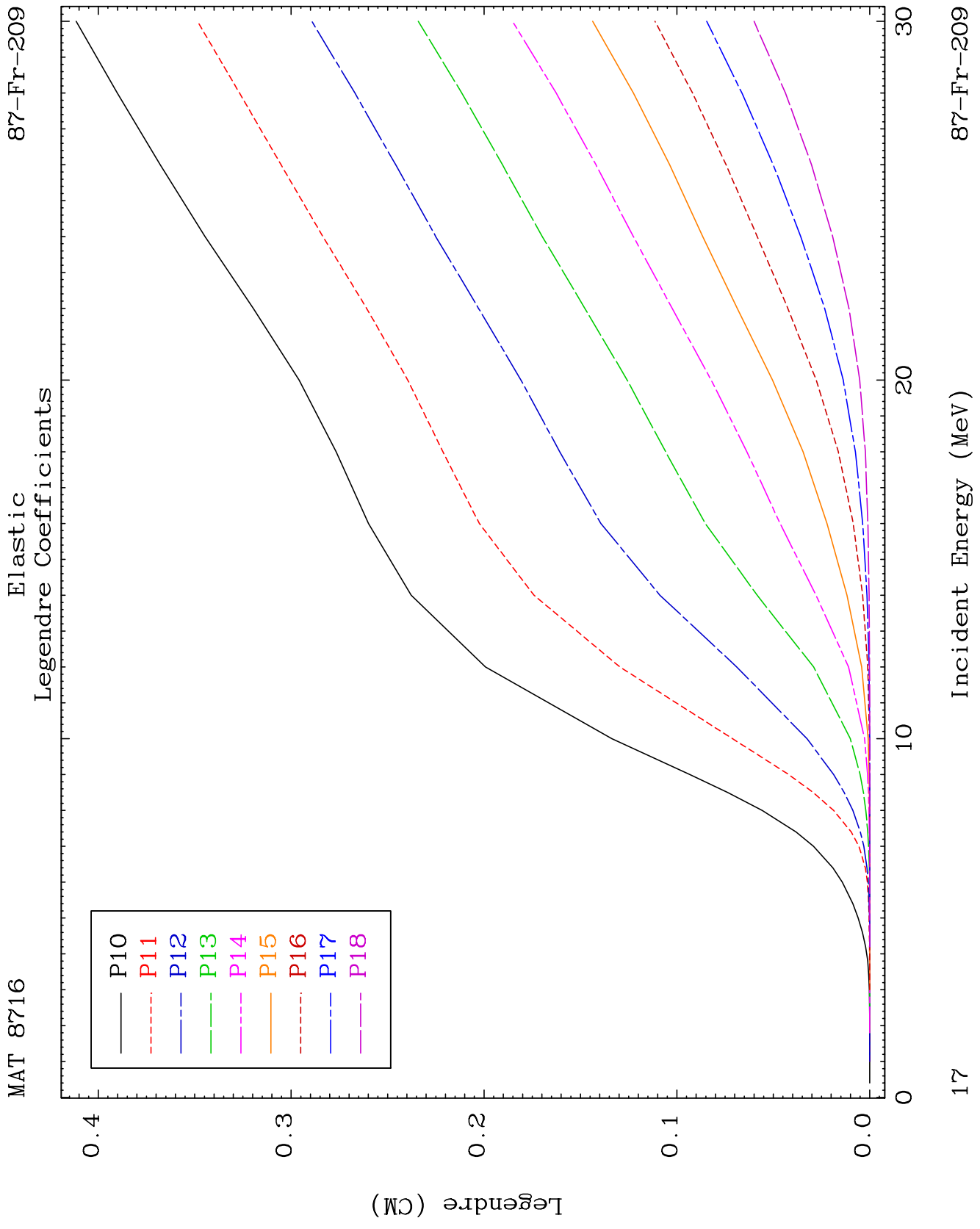
87-Fr-209



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Incident Energy (MeV)

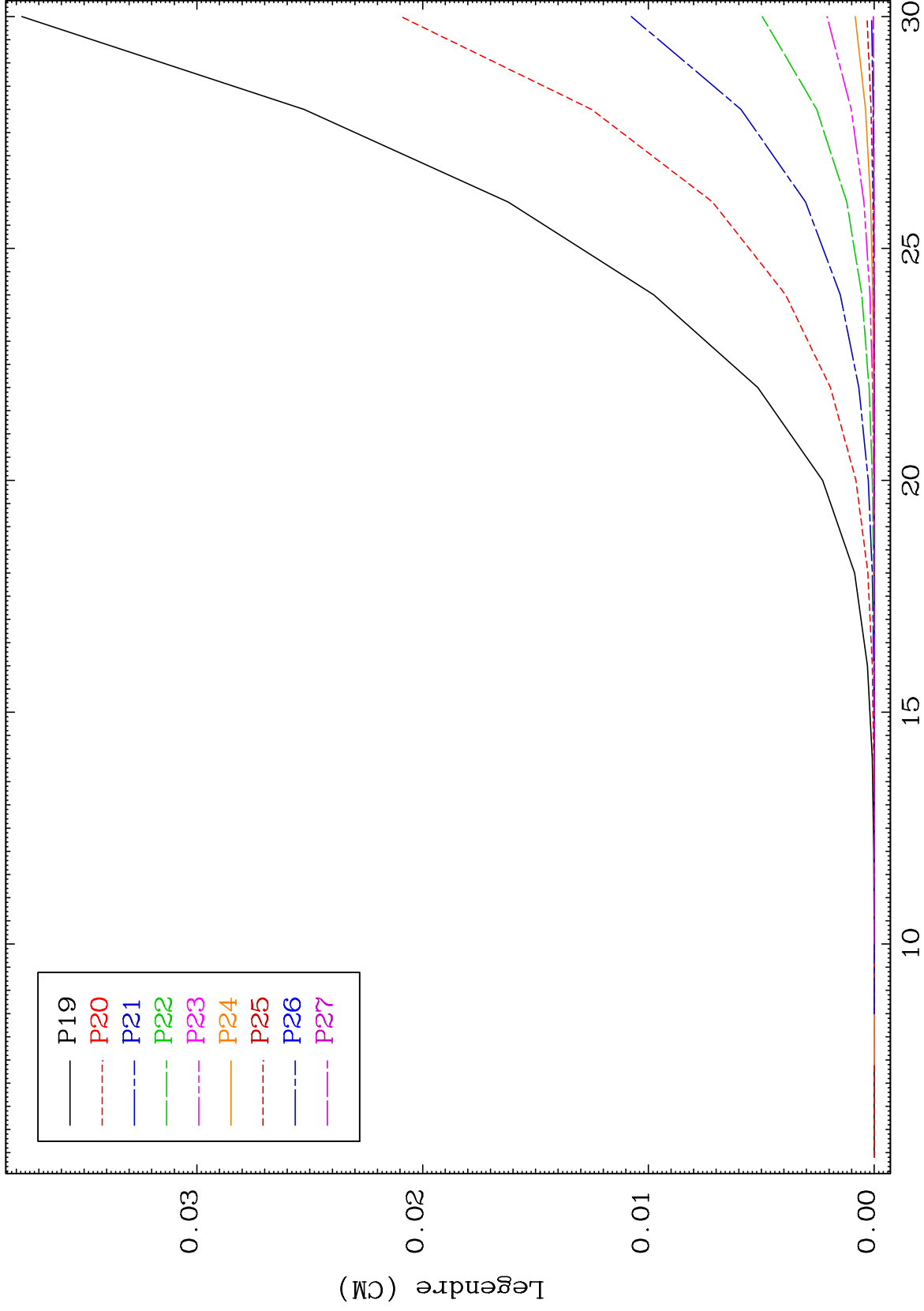
87-Fr-209



MAT 8716

Elastic Legendre Coefficients

87-Fr-209



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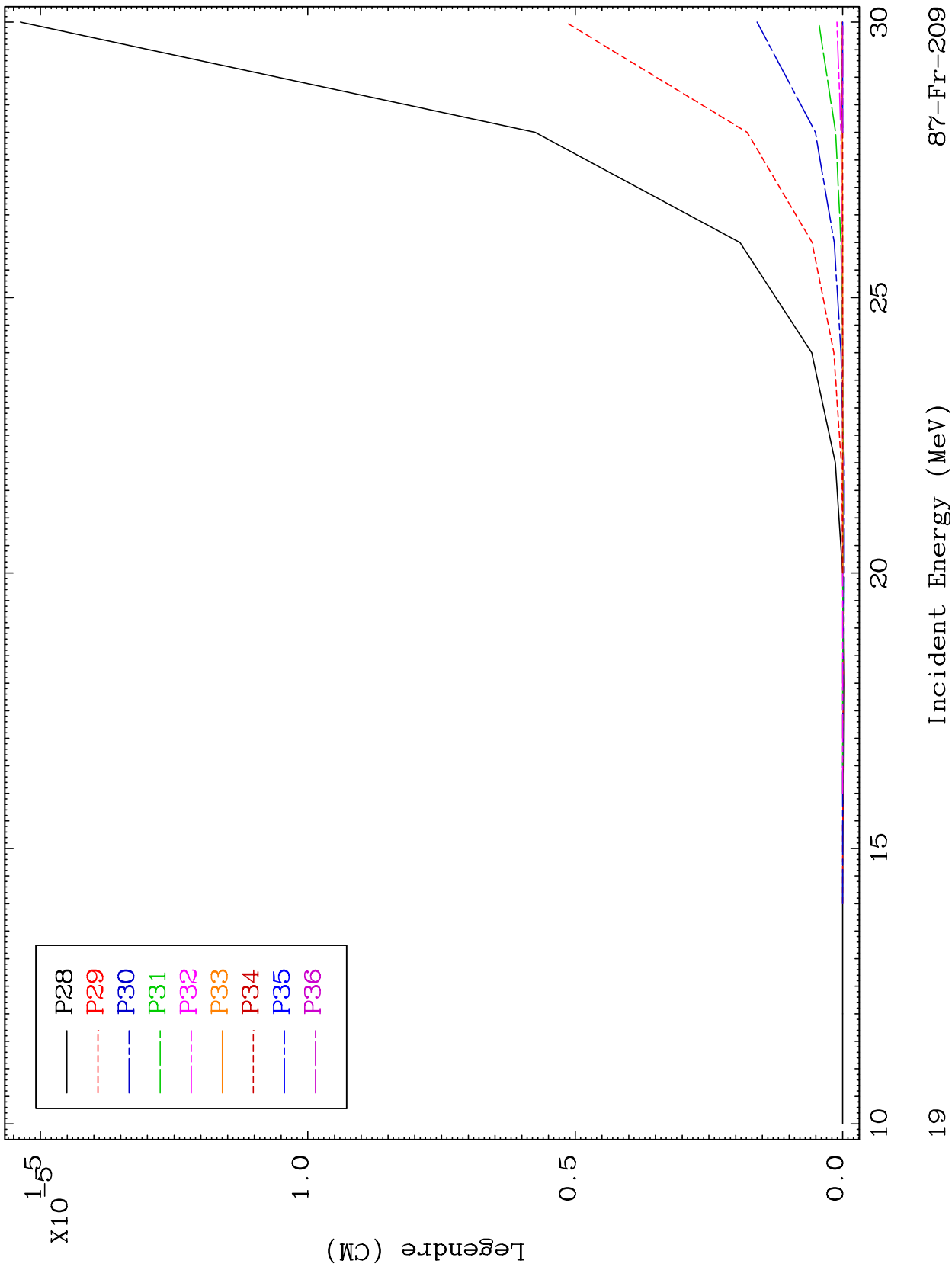
Incident Energy (MeV)

87-Fr-209

MAT 8716

Elastic
Legendre Coefficients

87-Fr-209



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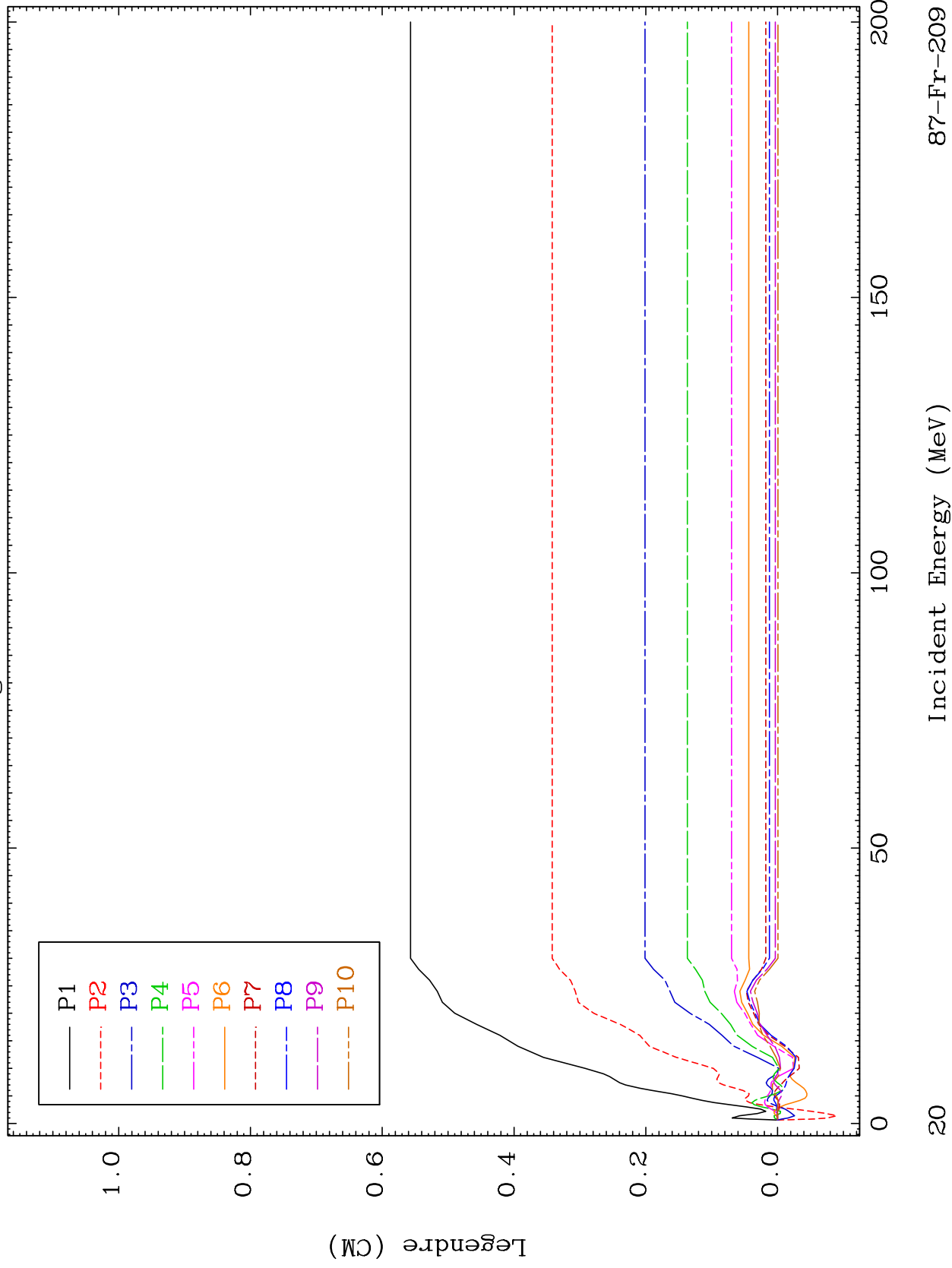
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 51 (n,n') Level
Legendre Coefficients

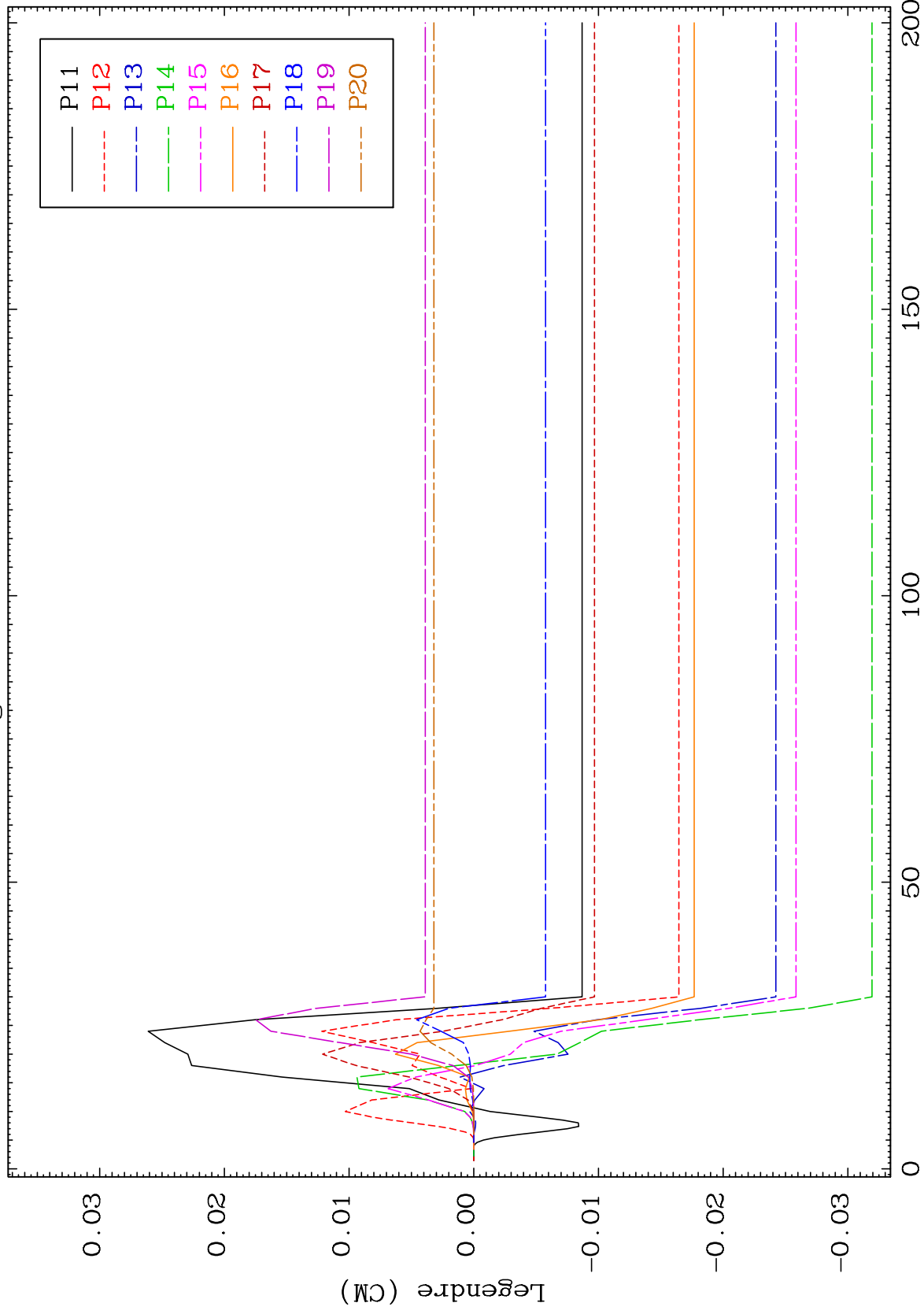
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MAT 8716

MT= 51 (n,n') Level
Legendre Coefficients

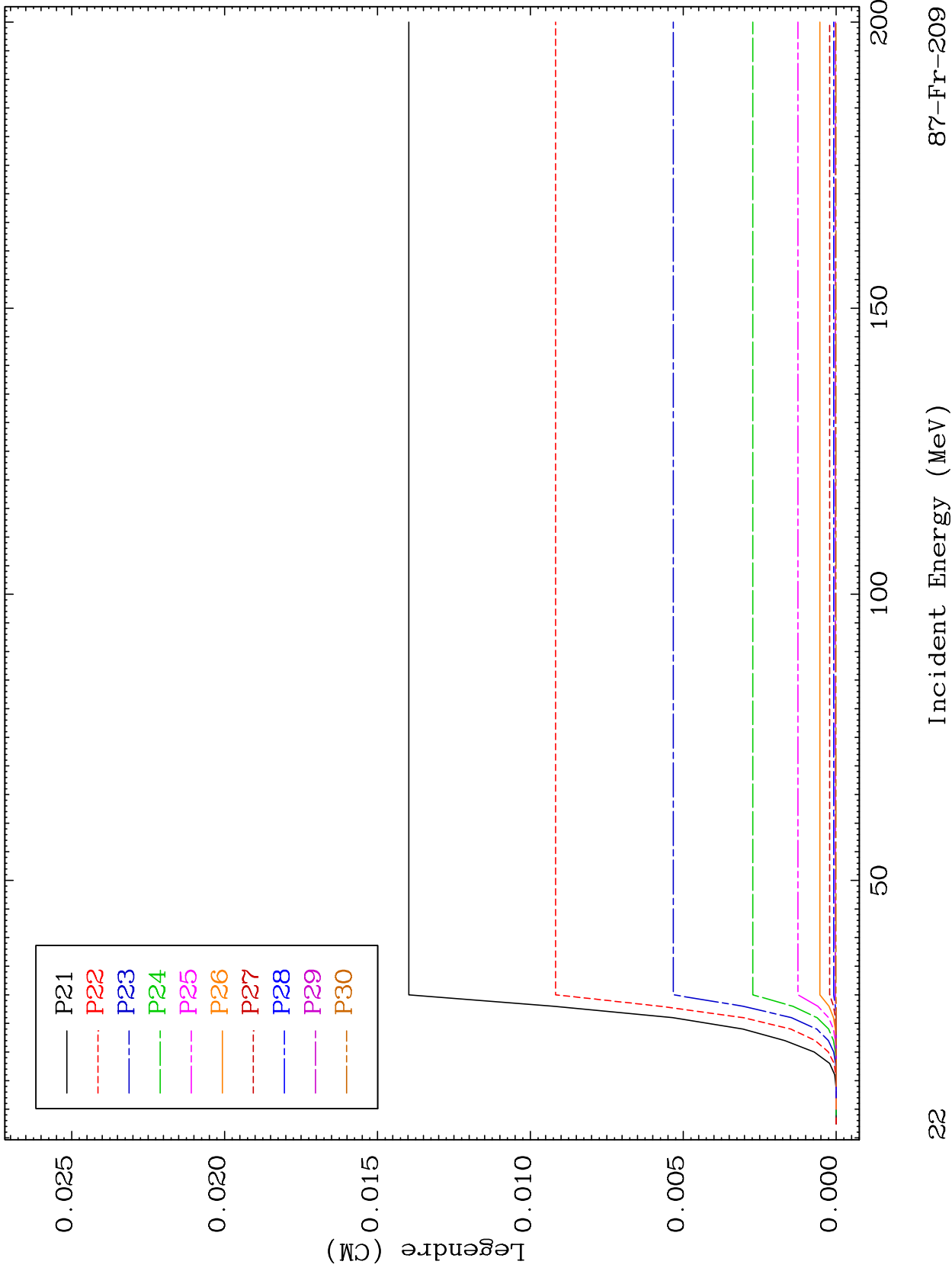
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Incident Energy (MeV)

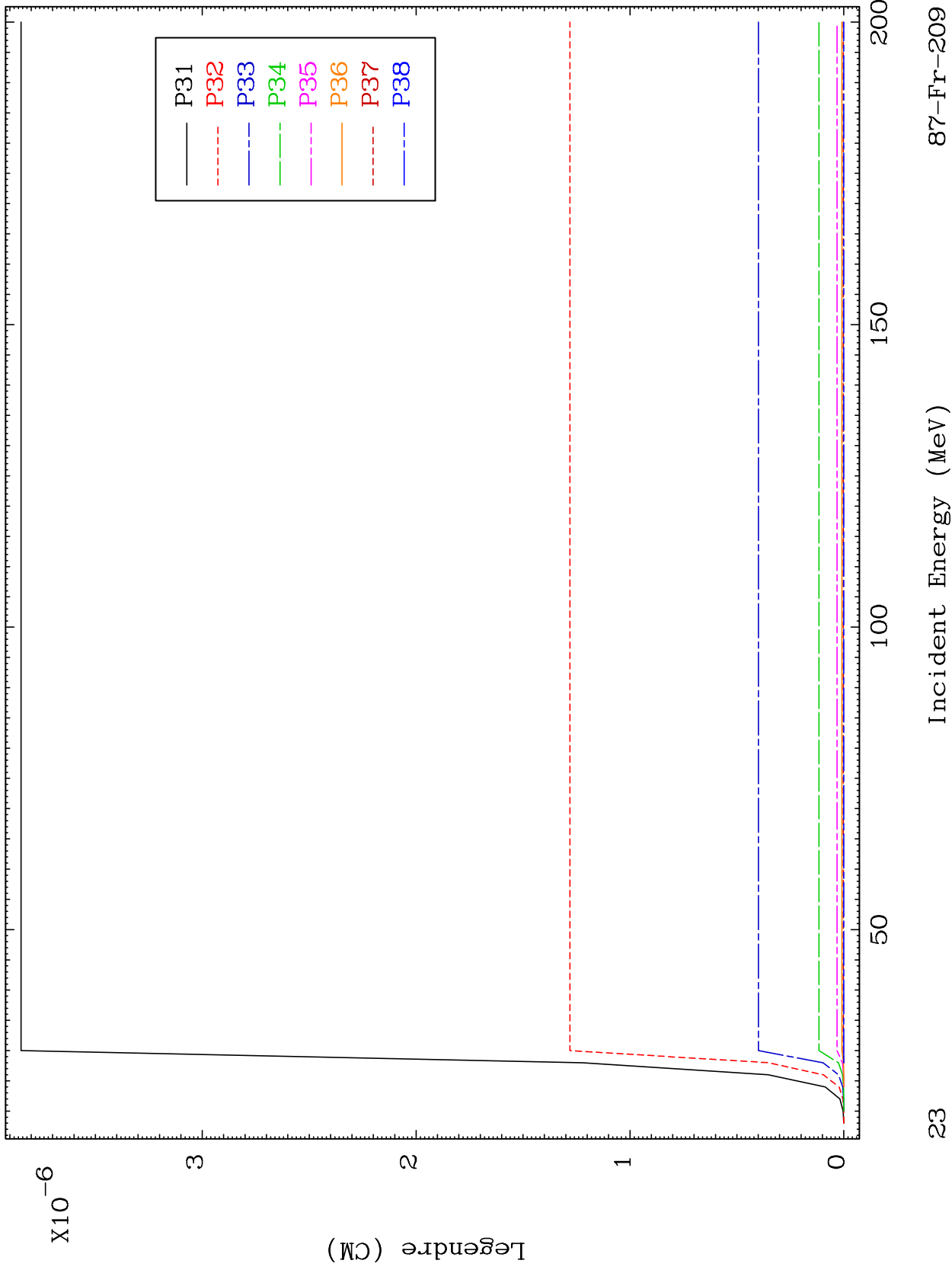
87-Fr-209



MAT 8716

MT= 51 (n,n') Level
Legendre Coefficients

87-Fr-209



23

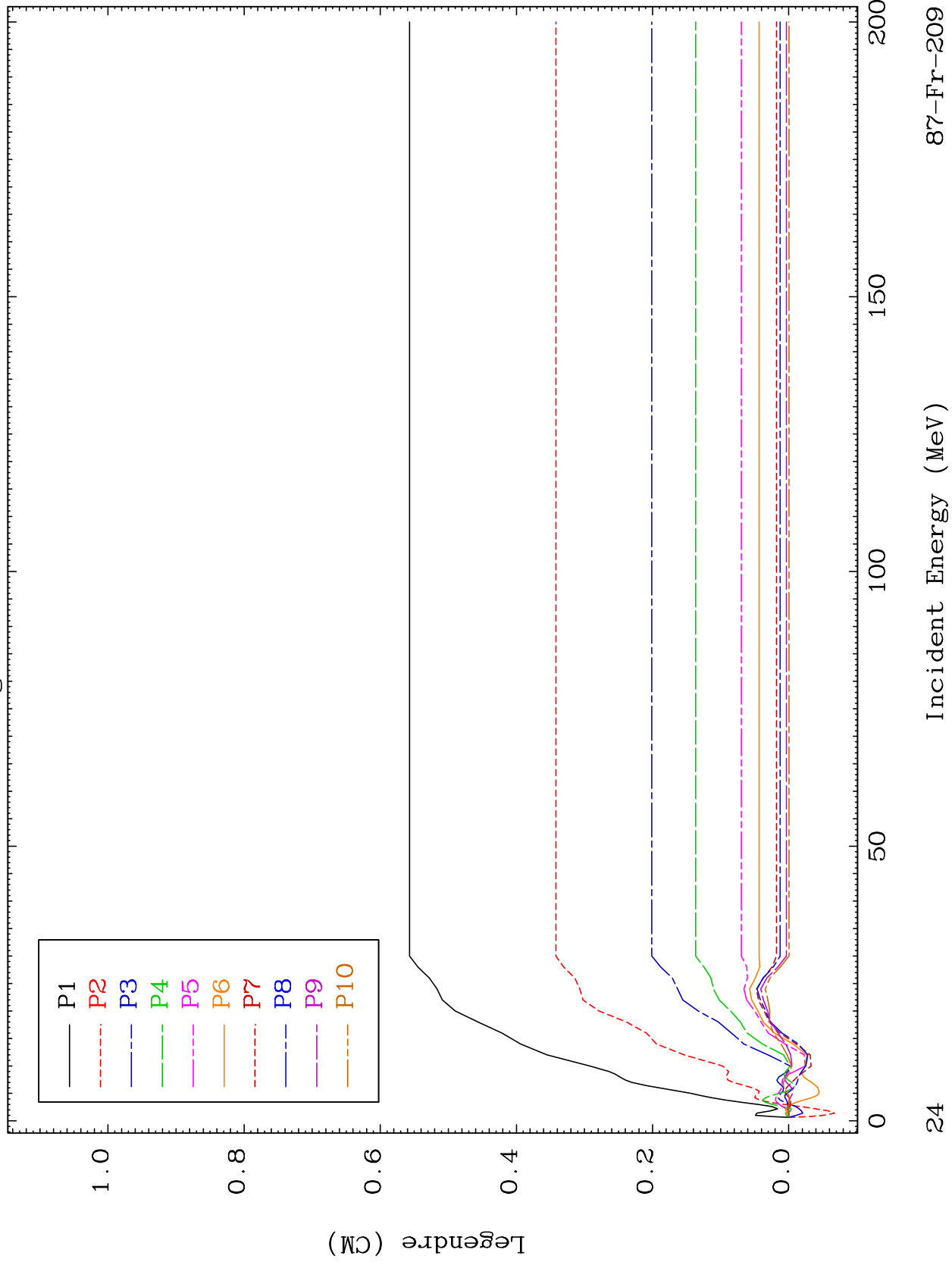
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 52 (n,n') Level
Legendre Coefficients

87-Fr-209



87-Fr-209

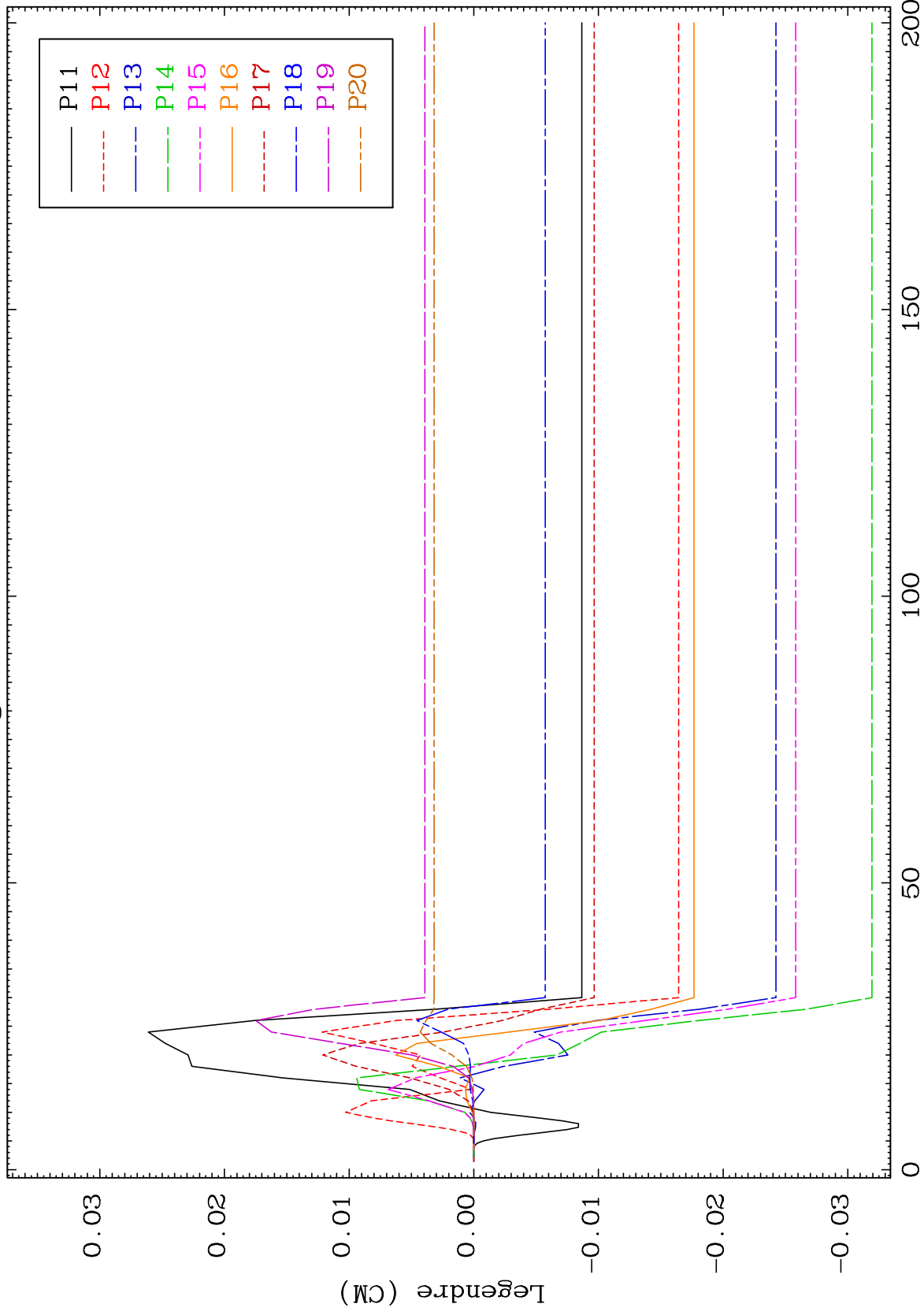
Incident Energy (MeV)

24

MAT 8716

MT= 52 (n,n') Level
Legendre Coefficients

87-Fr-209



87-Fr-209

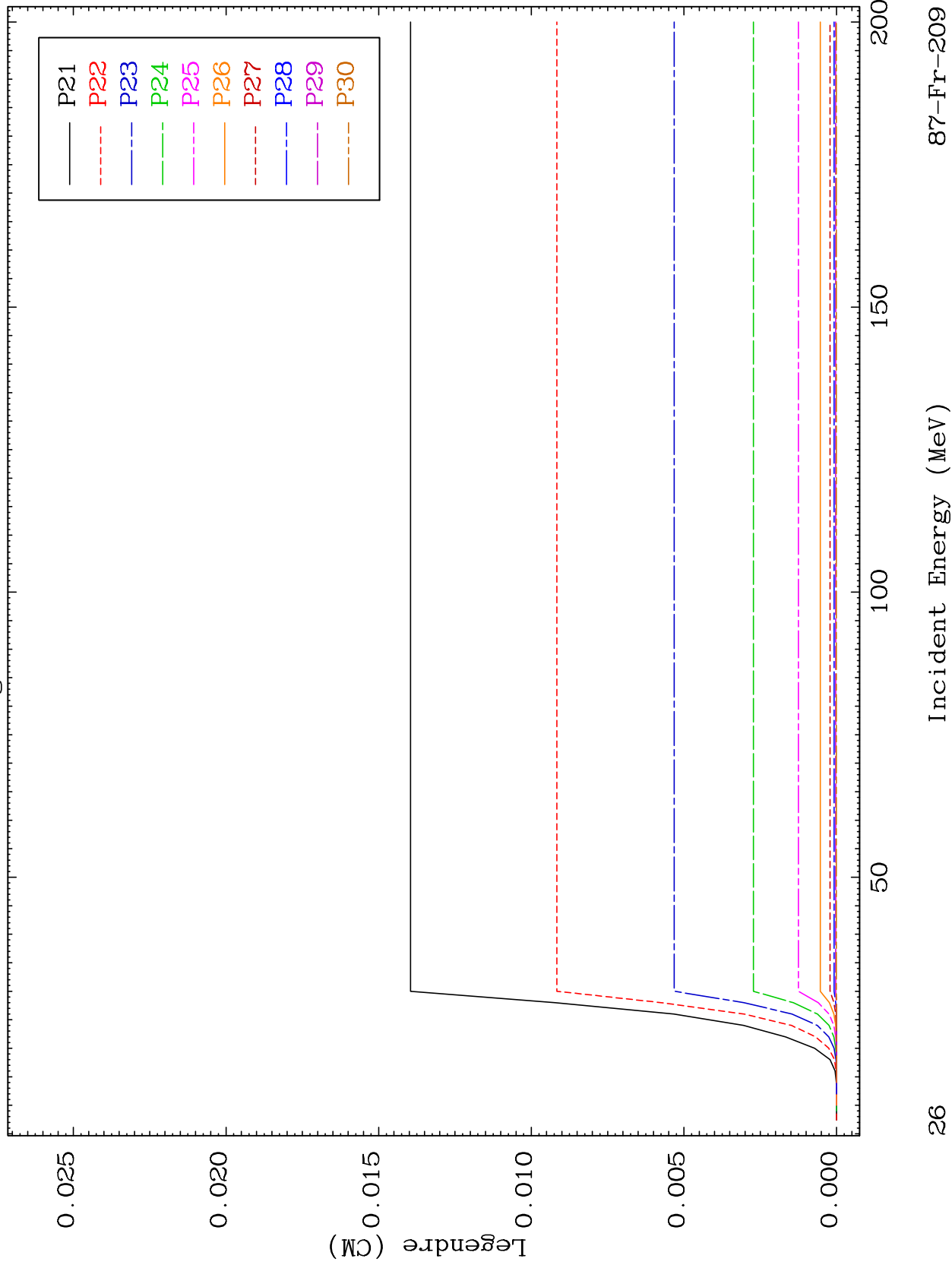
Incident Energy (MeV)

25

MAT 8716

MT= 52 (n,n') Level
Legendre Coefficients

87-Fr-209



26

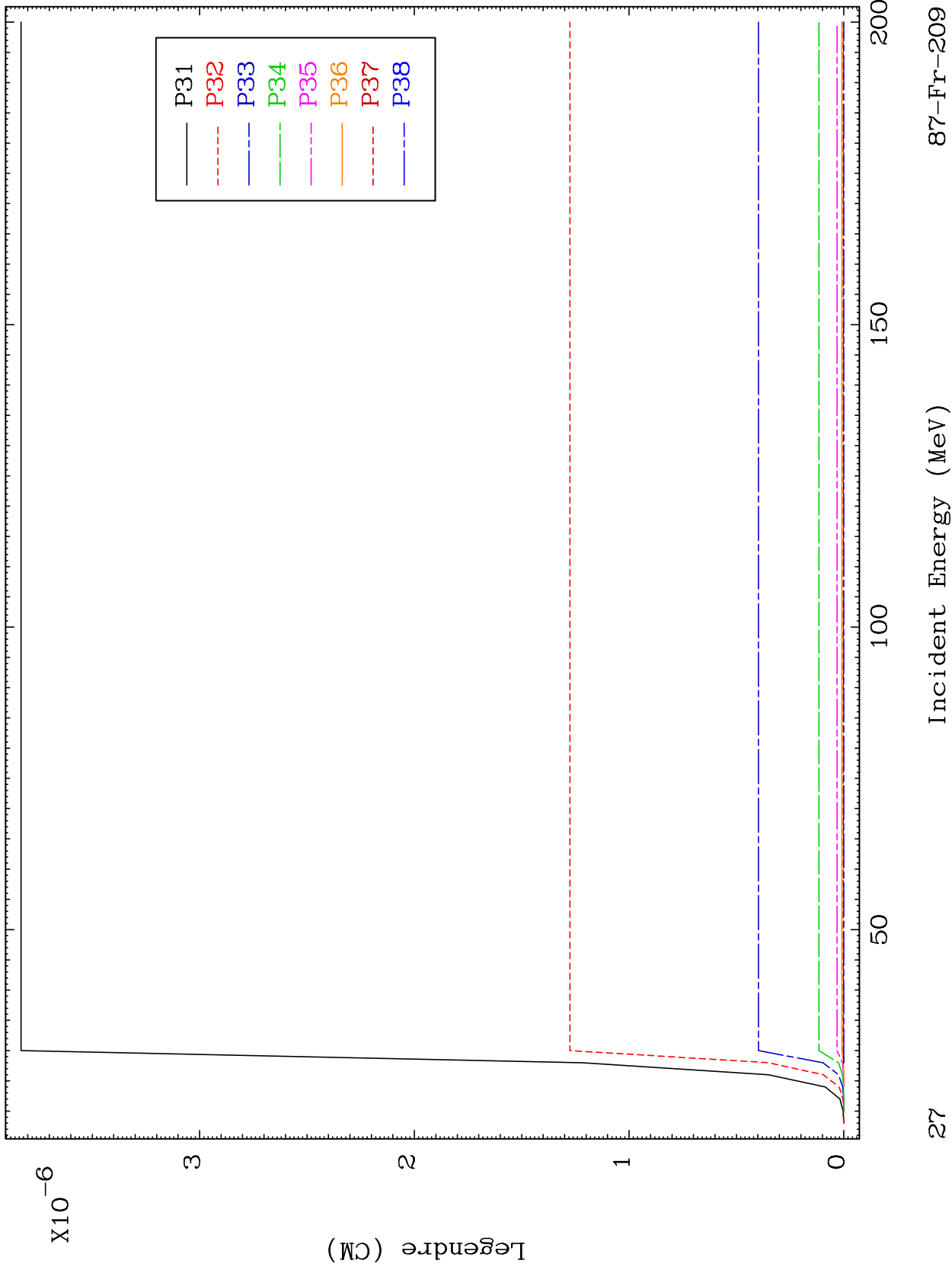
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 52 (n,n') Level
Legendre Coefficients

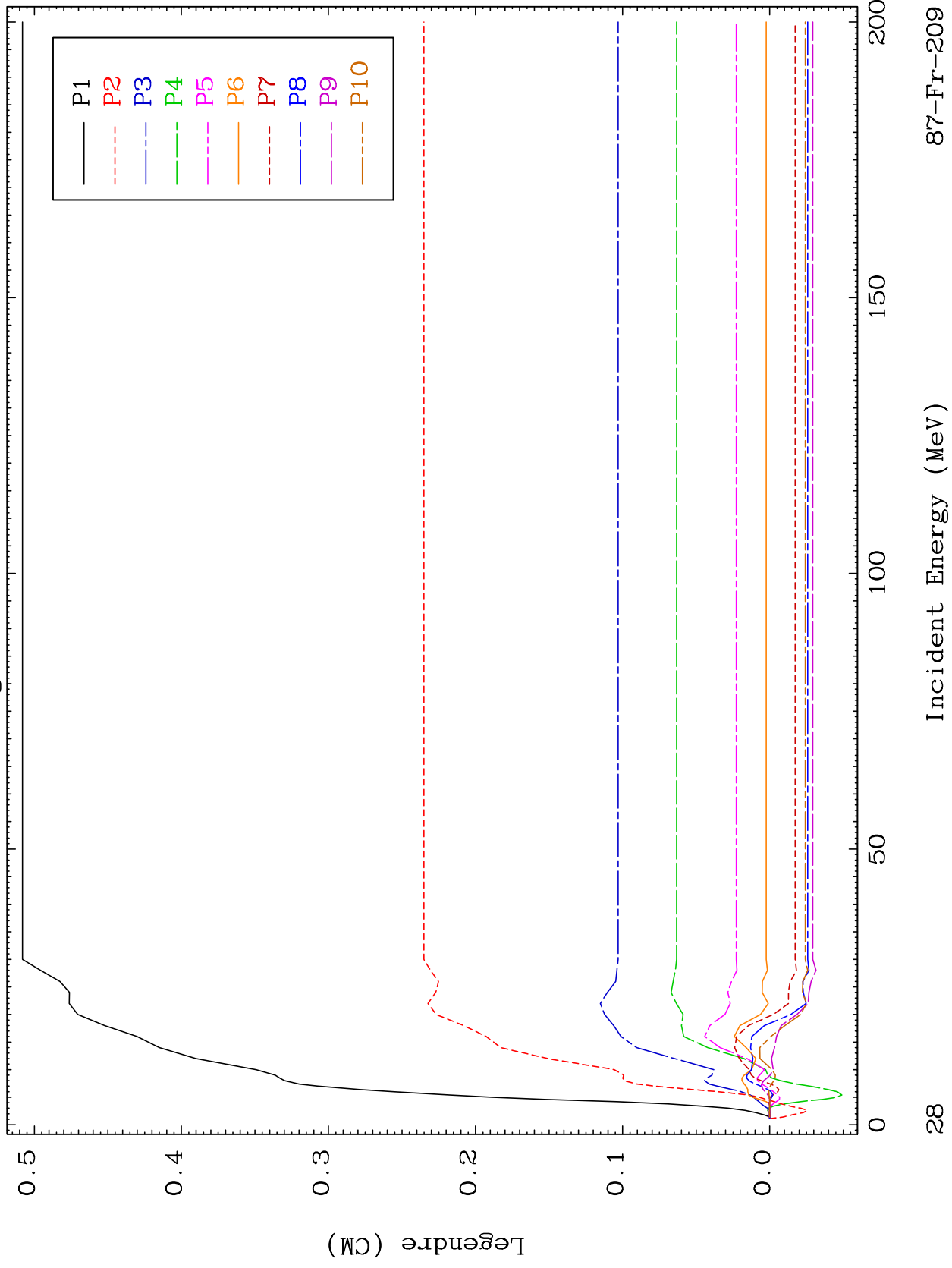
87-Fr-209



MAT 8716

MT= 53 (n,n') Level
Legendre Coefficients

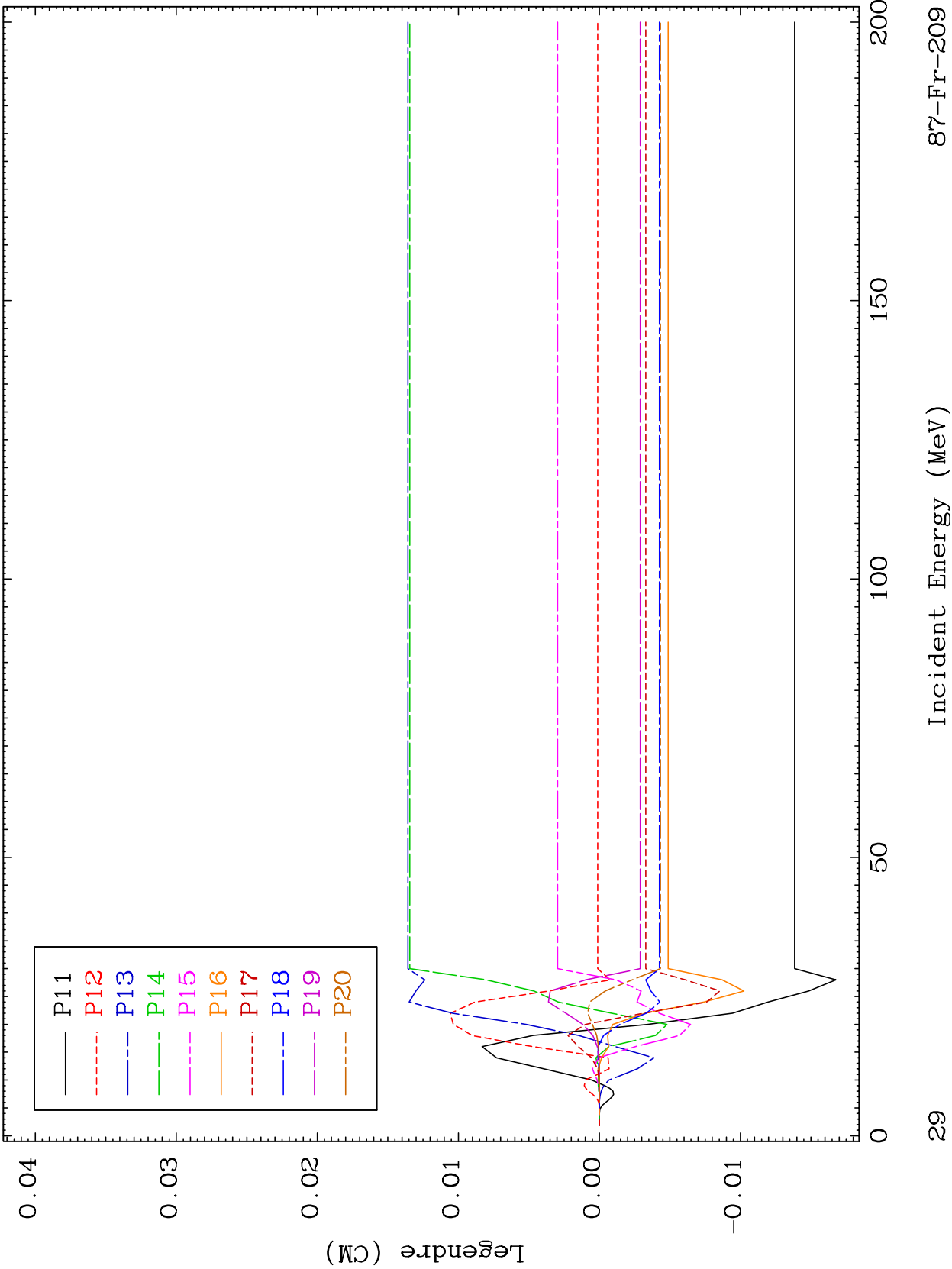
87-Fr-209



MAT 8716

MT= 53 (n,n') Level
Legendre Coefficients

87-Fr-209



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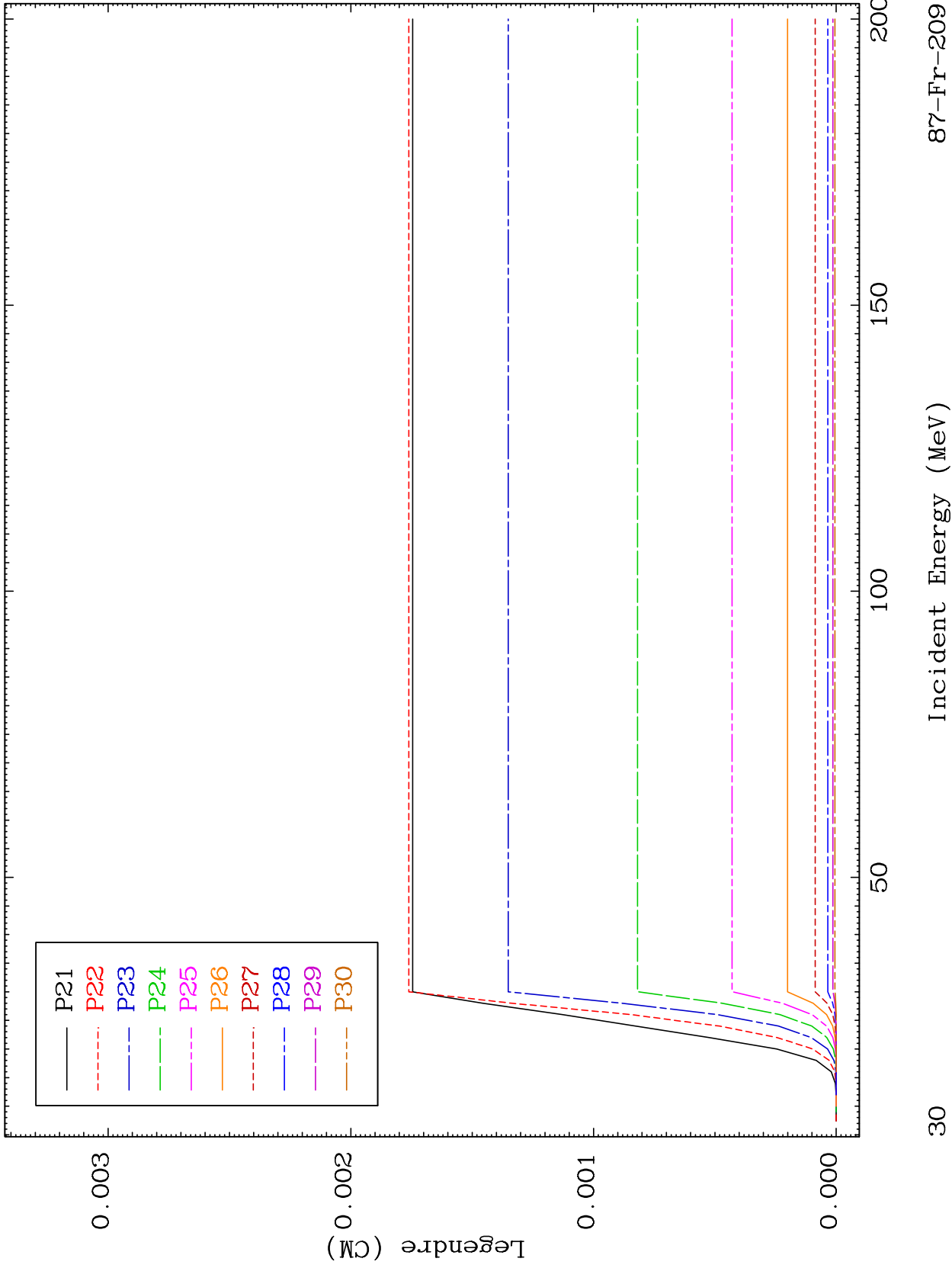
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 53 (n,n') Level
Legendre Coefficients

87-Fr-209



30

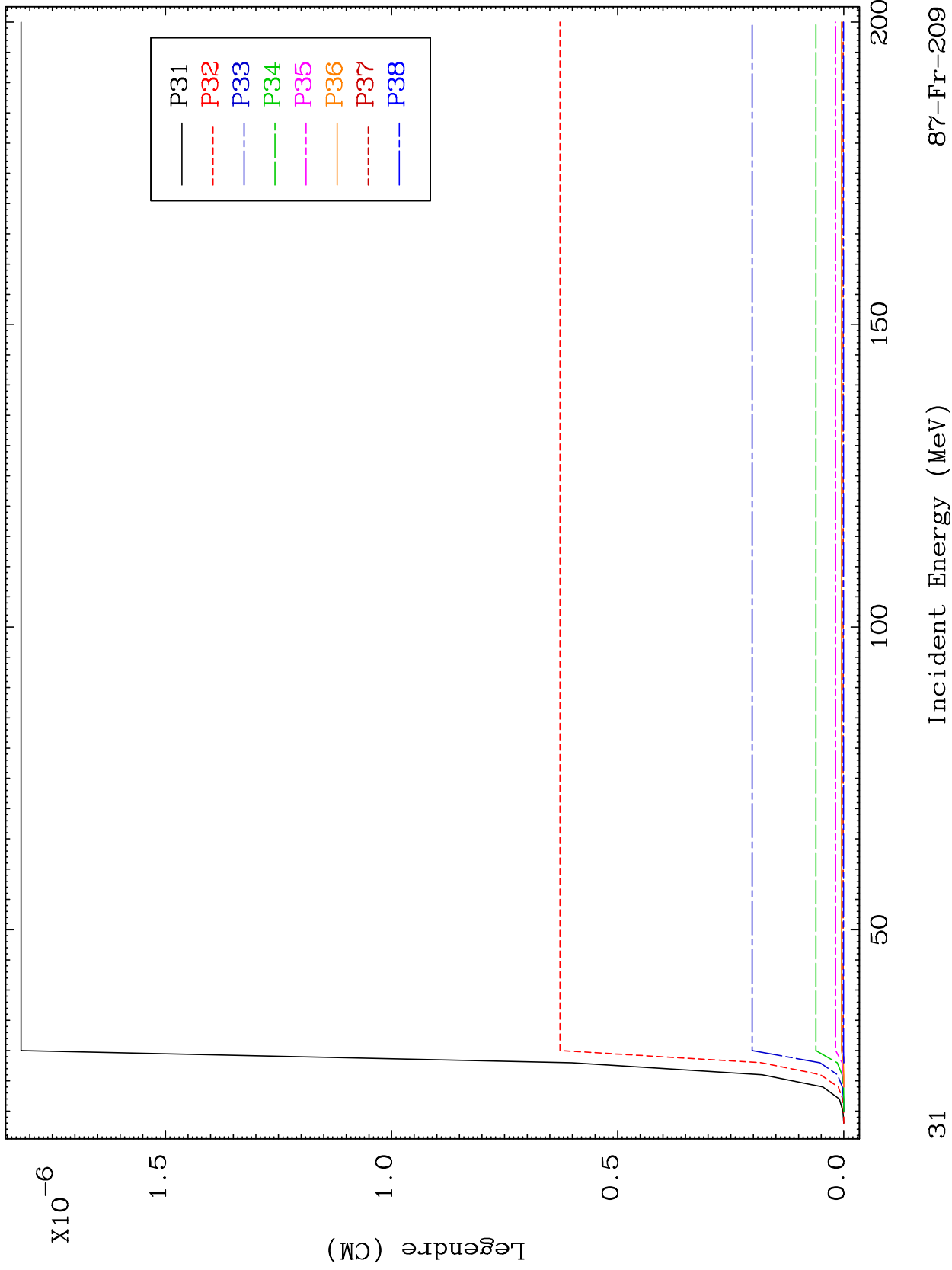
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 53 (n,n') Level
Legendre Coefficients

87-Fr-209

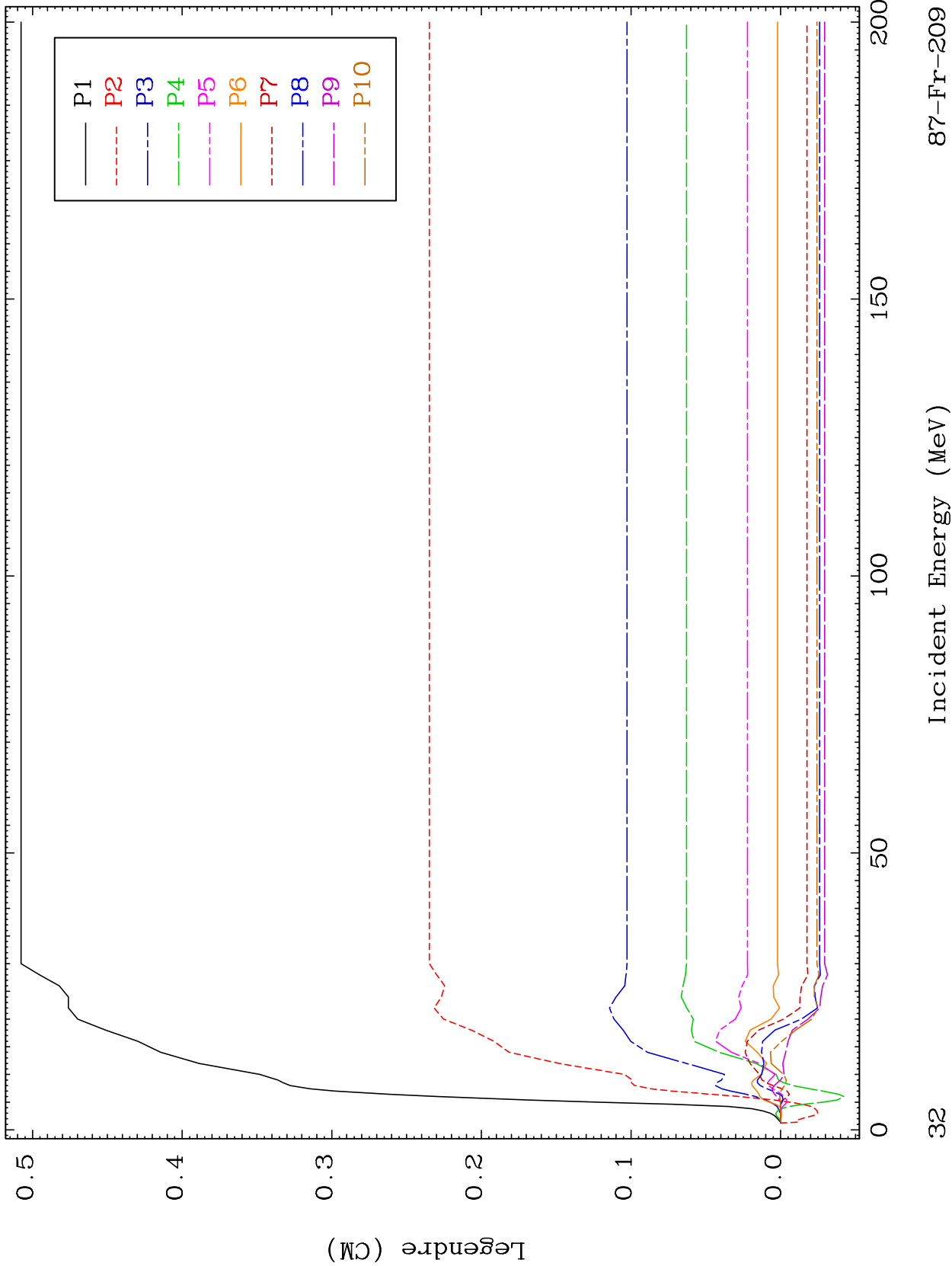


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MAT 8716

MT= 54 (n,n') Level
Legendre Coefficients

87-Fr-209



87-Fr-209

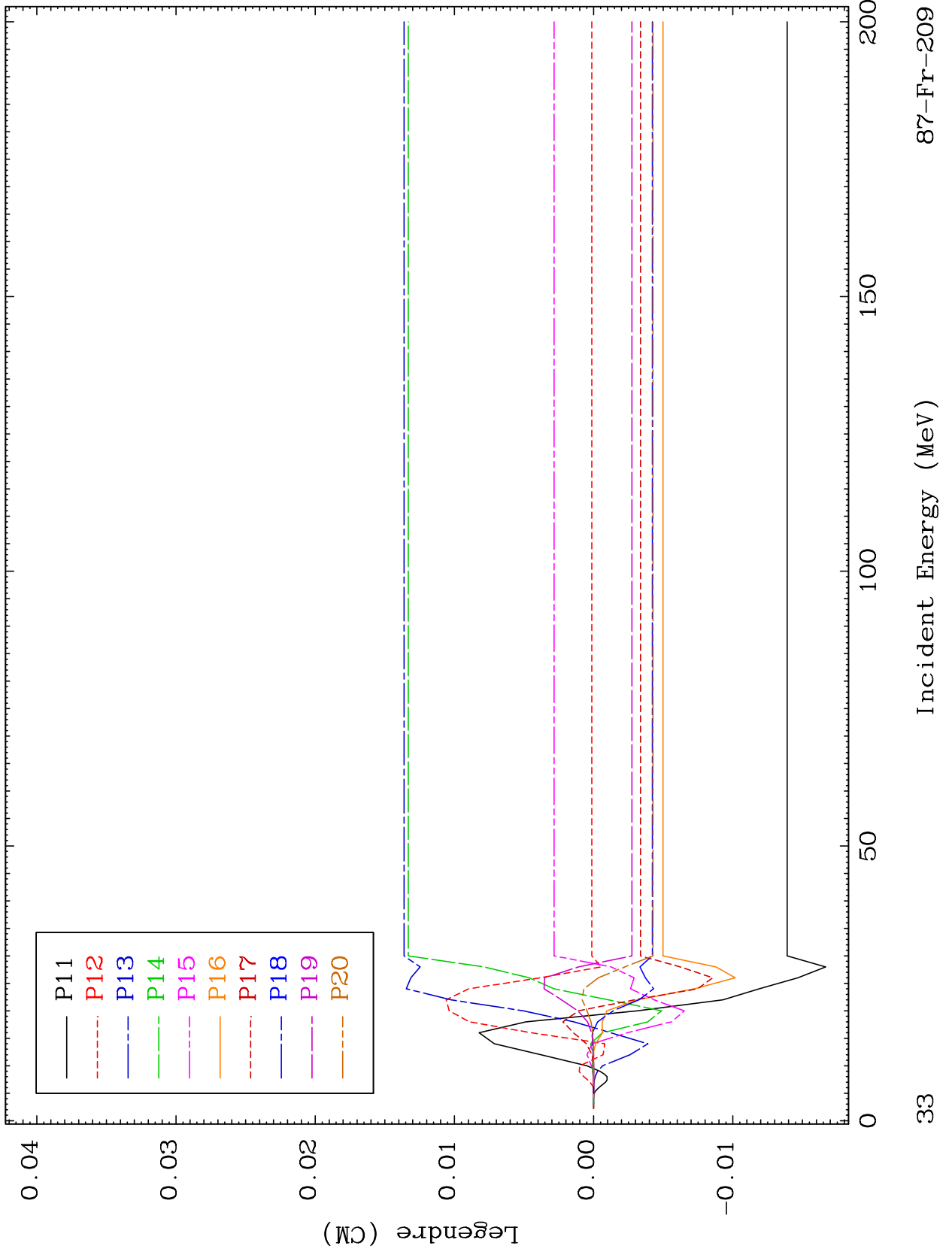
Incident Energy (MeV)

32

MAT 8716

MT= 54 (n,n') Level
Legendre Coefficients

87-Fr-209



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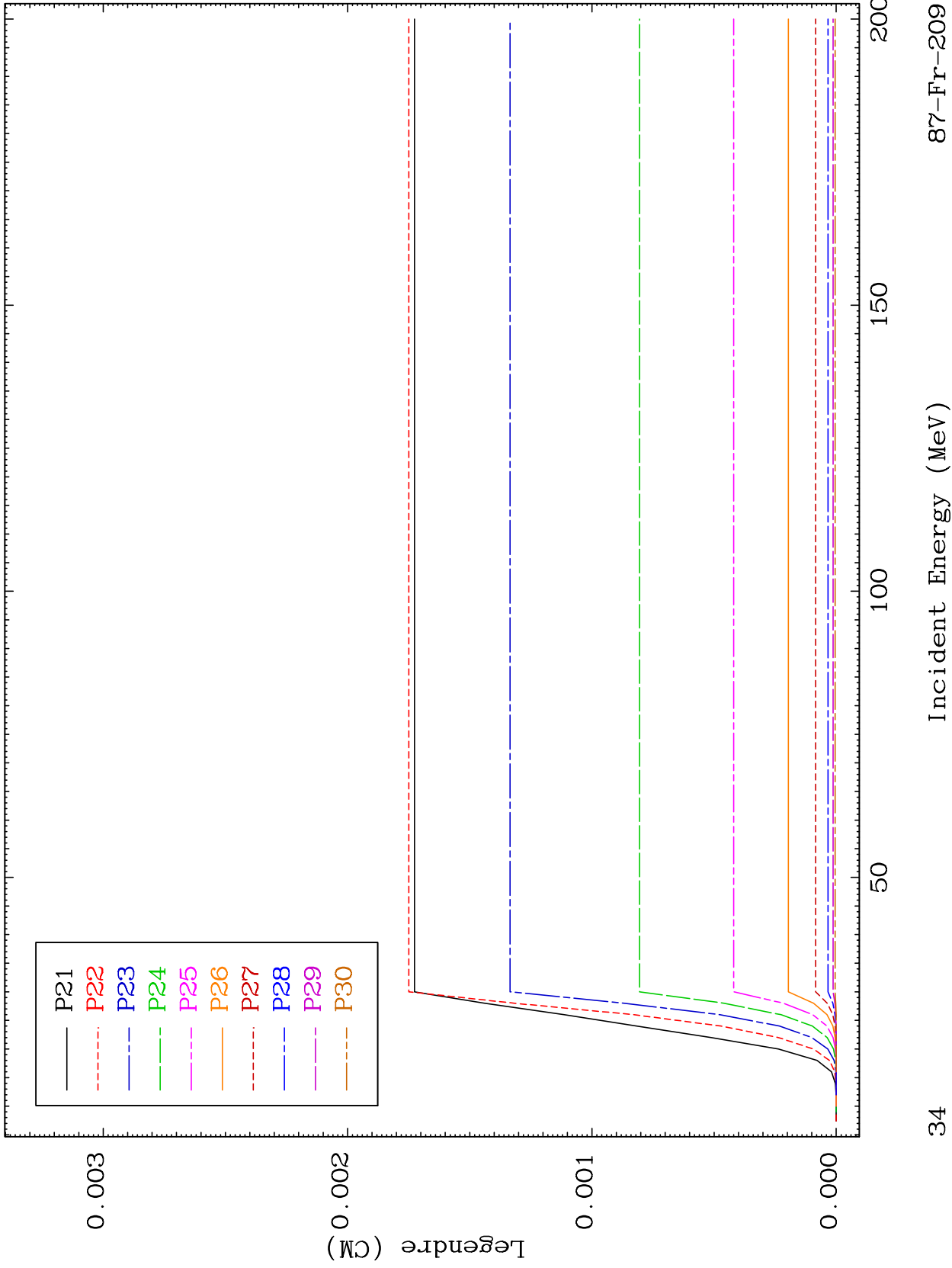
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 54 (n,n') Level
Legendre Coefficients

87-Fr-209



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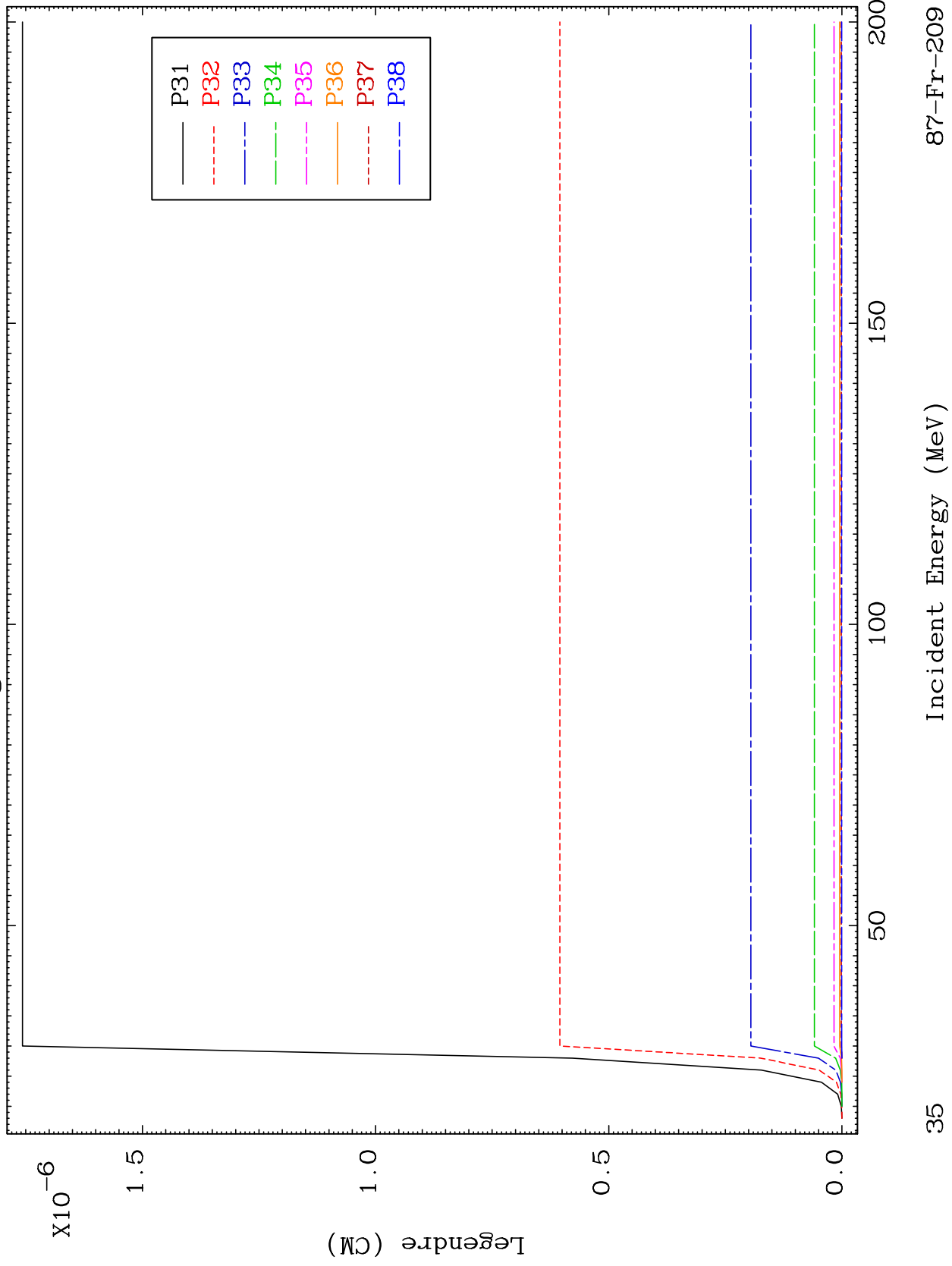
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 54 (n,n') Level
Legendre Coefficients

87-Fr-209

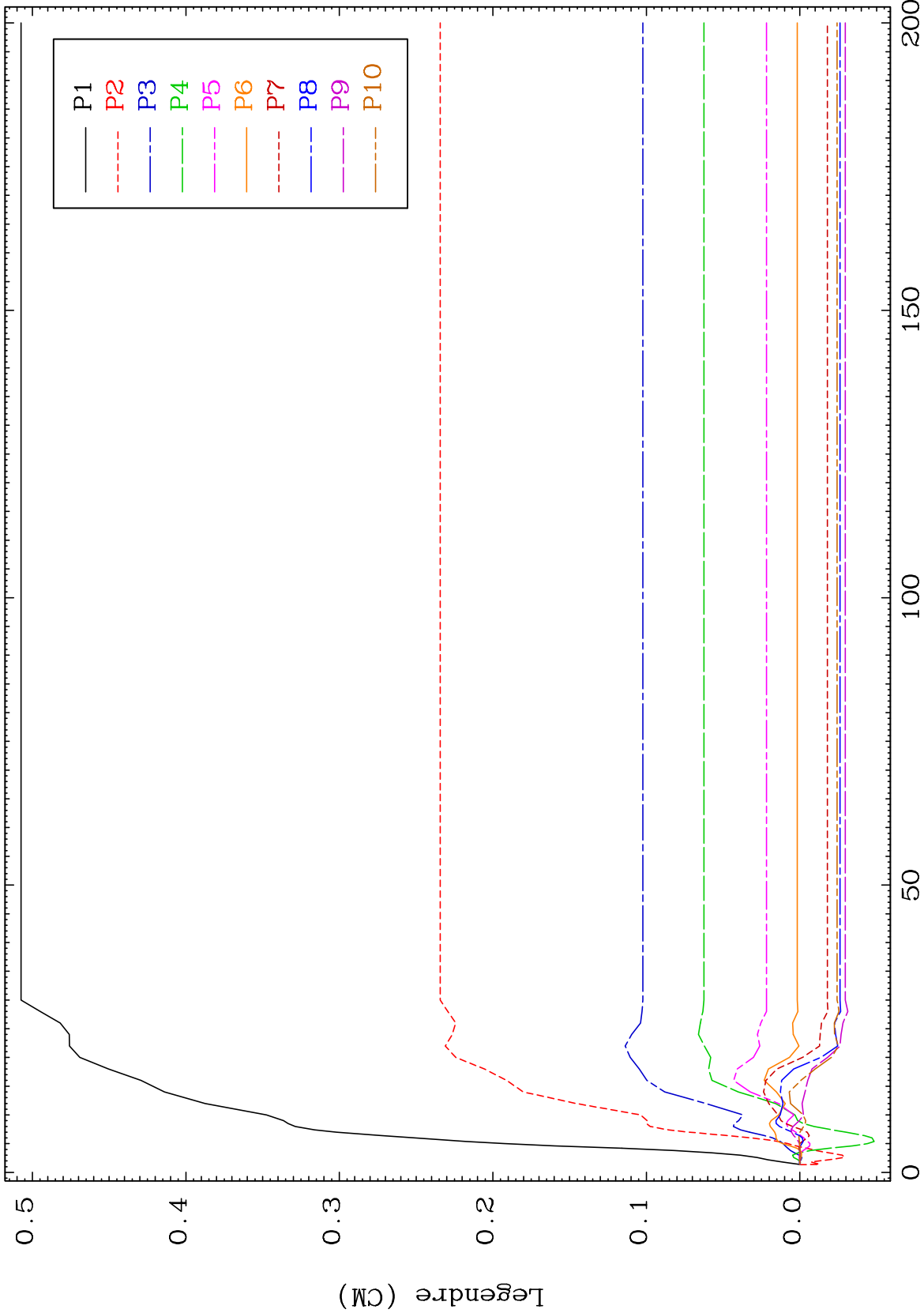


MAT 8716

MT= 55 (n,n') Level

87-Fr-209

Legendre Coefficients



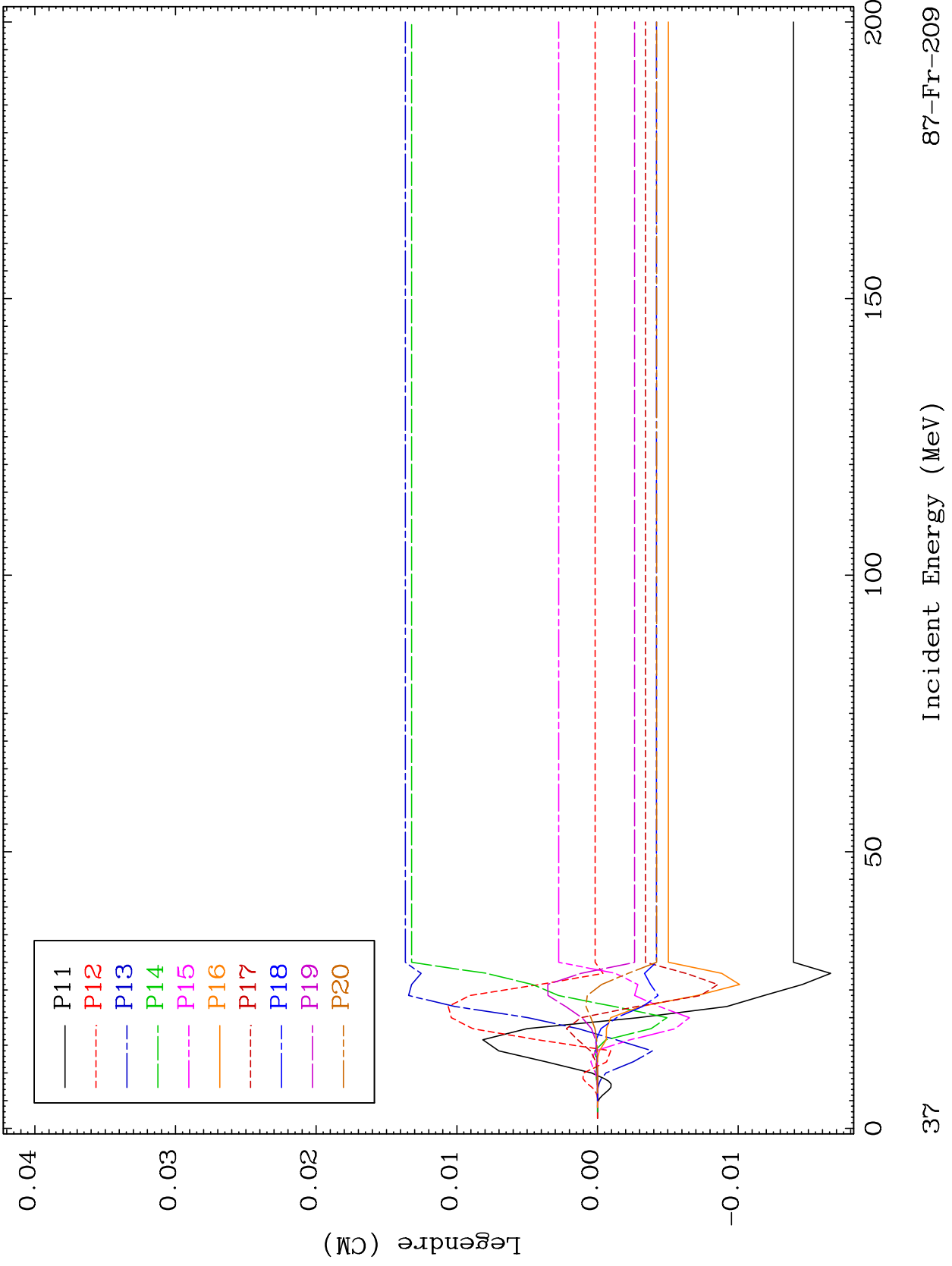
36

87-Fr-209

MAT 8716

MT= 55 (n,n') Level
Legendre Coefficients

87-Fr-209



37

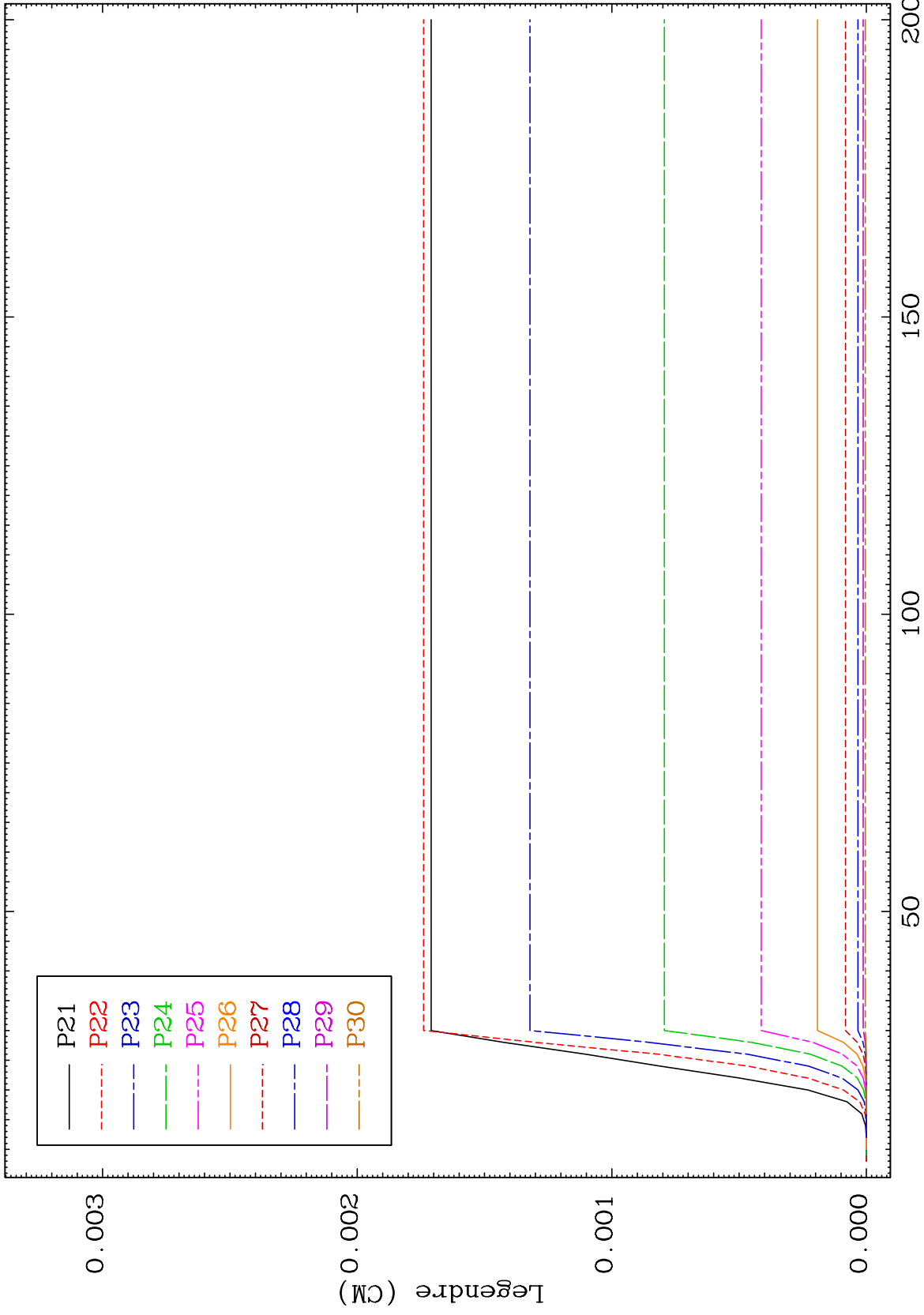
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 55 (n,n') Level
Legendre Coefficients

87-Fr-209



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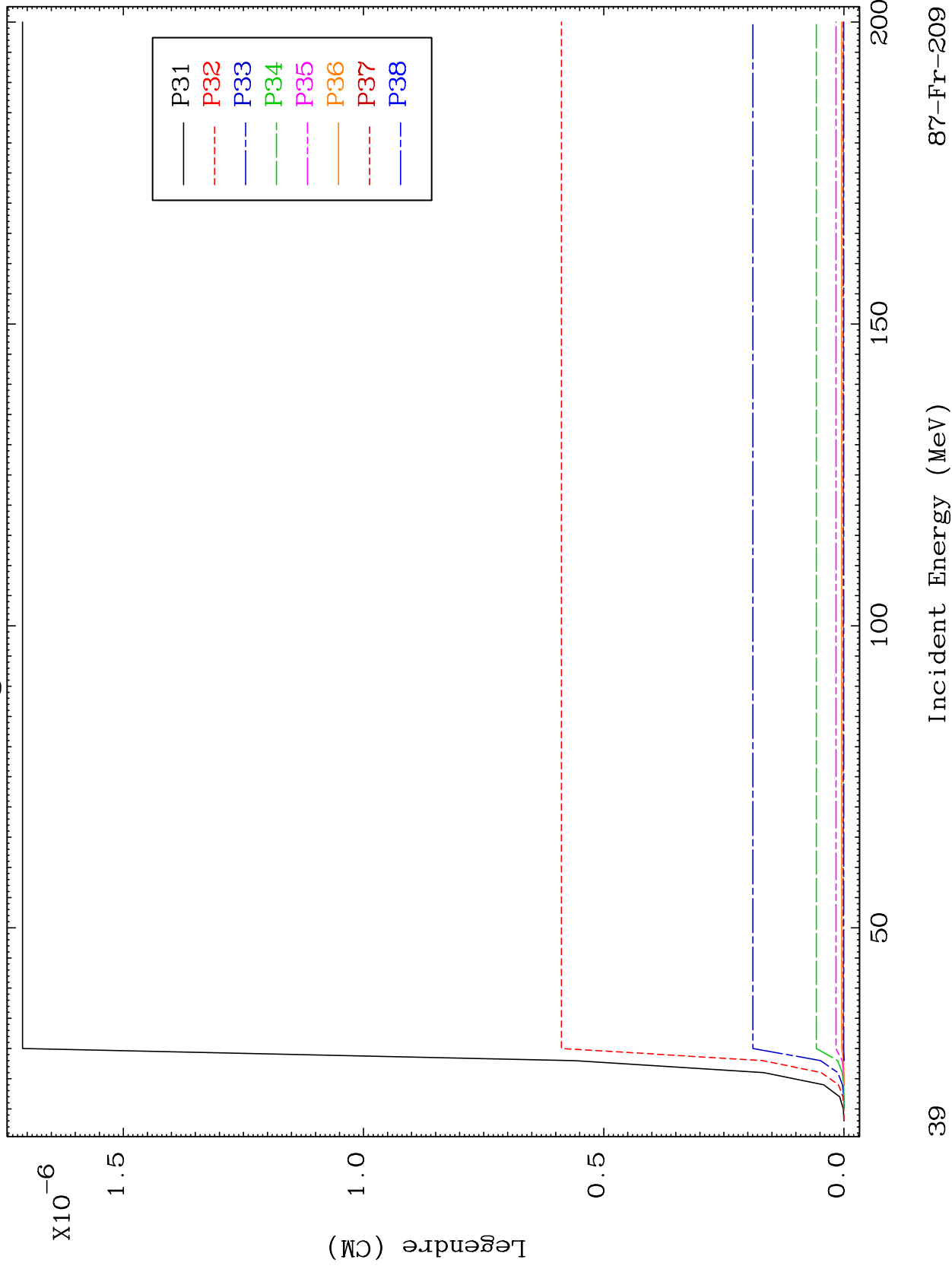
Incident Energy (MeV)

87-Fr-209

MAT 8716

MT= 55 (n,n') Level
Legendre Coefficients

87-Fr-209



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Incident Energy (MeV)

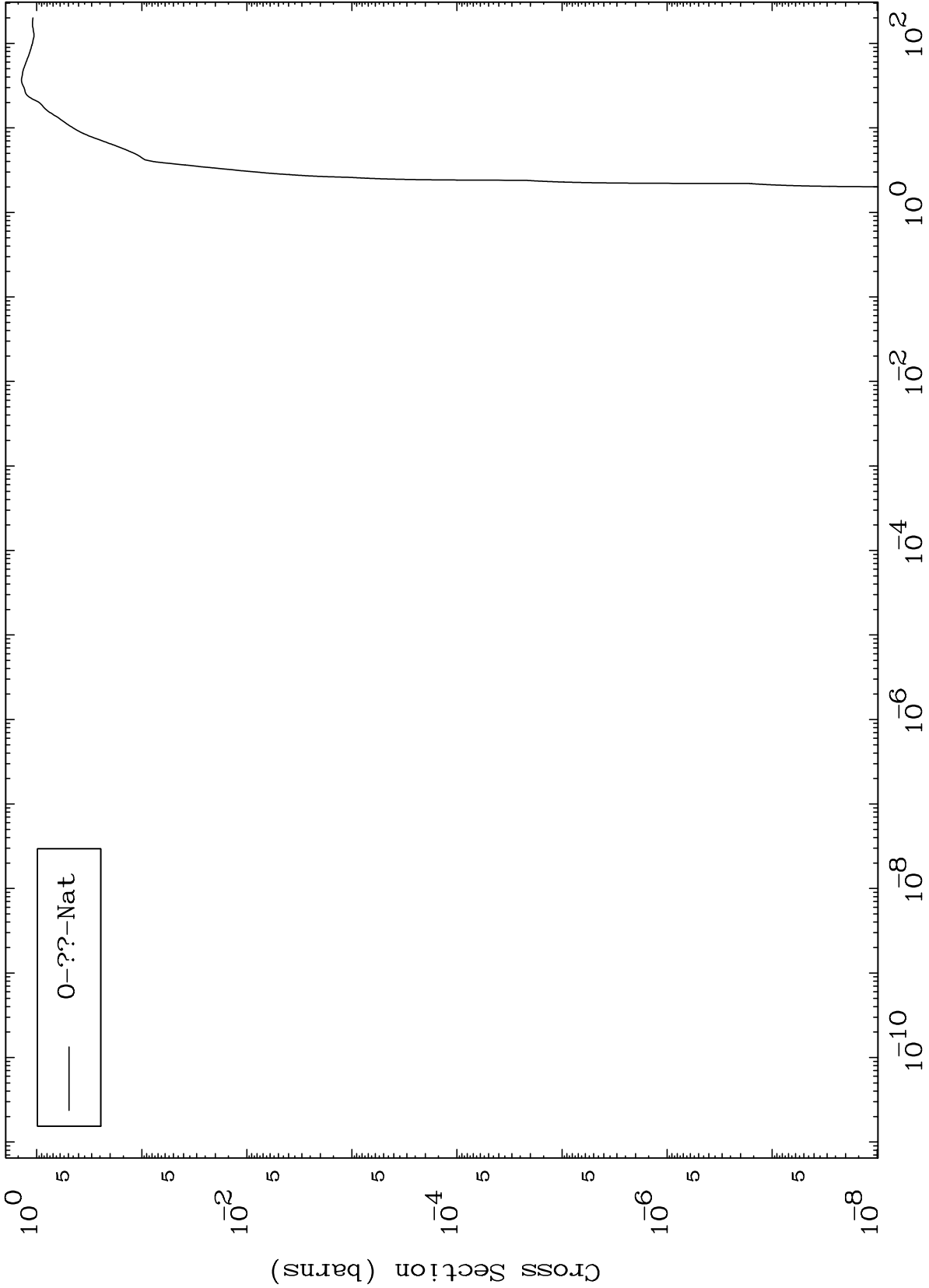
87-Fr-209

MAT 8716

Fission

87-Fr-209

Radionuclide Production Cross Section



40

Incident Energy (MeV)

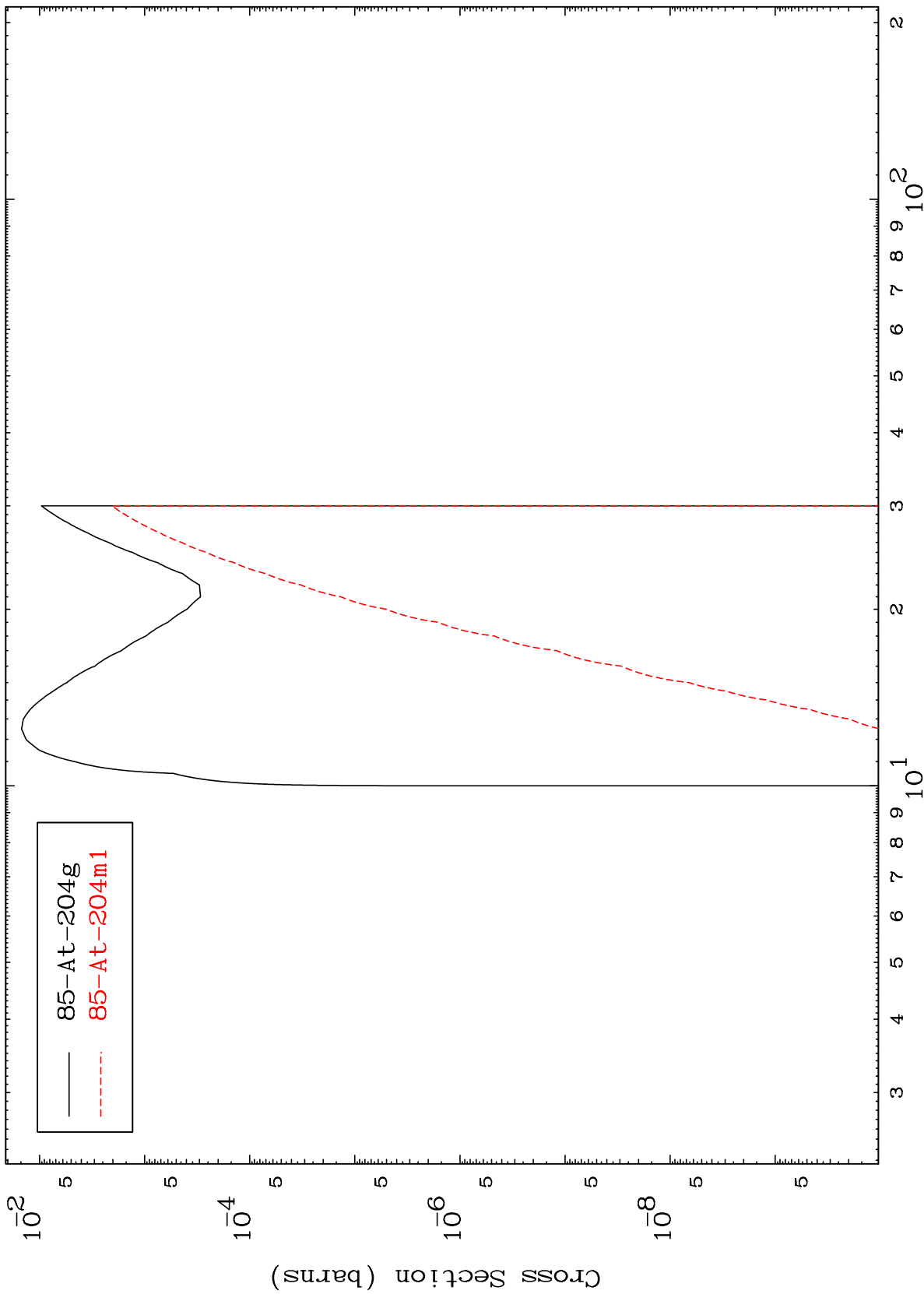
87-Fr-209

MAT 8716

(n,2n) α

87-Fr-209

Radionuclide Production Cross Section

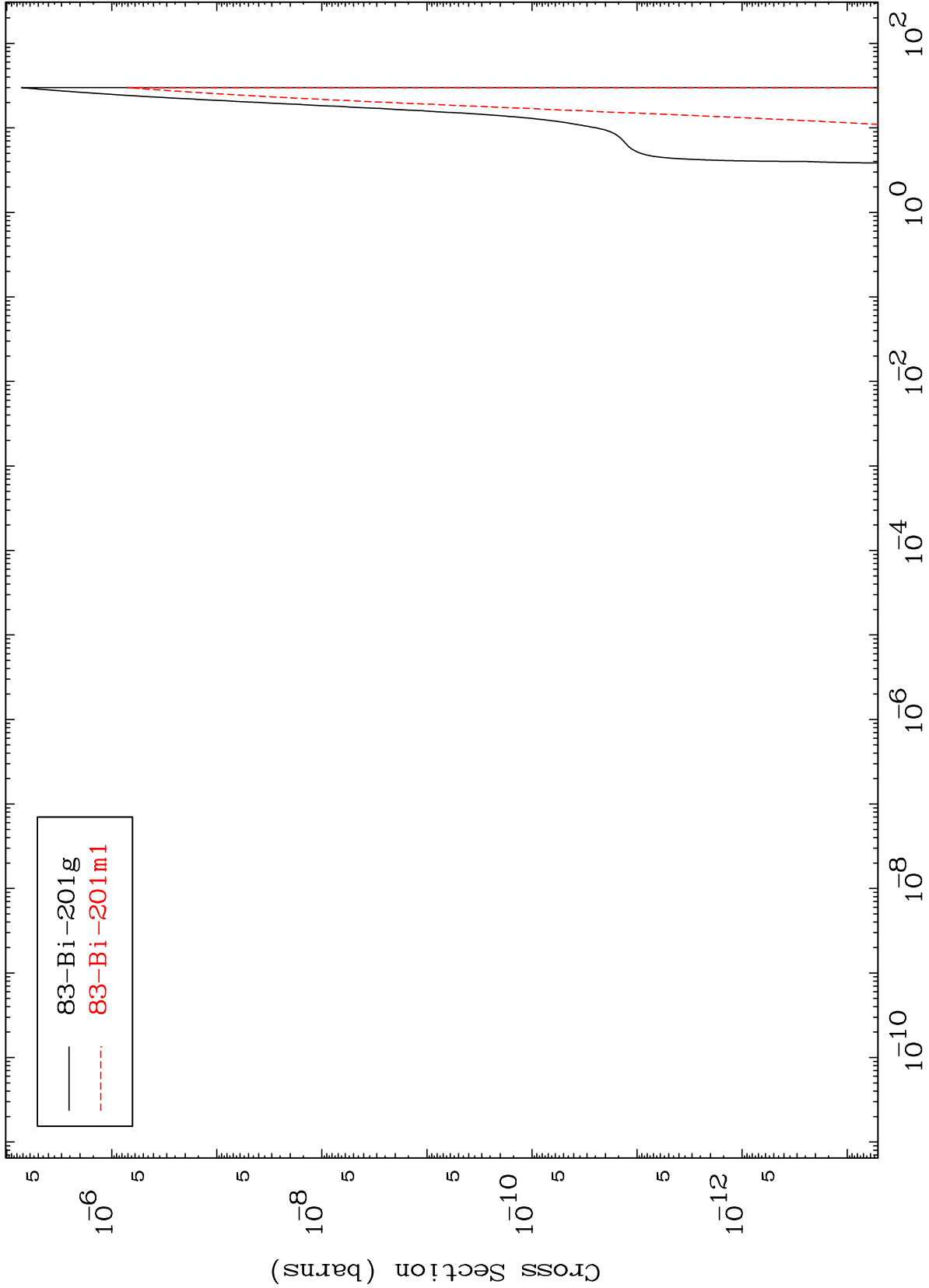


MAT 8716

(n,n') 2α

87-Fr-209

Radionuclide Production Cross Section



42

Incident Energy (MeV)

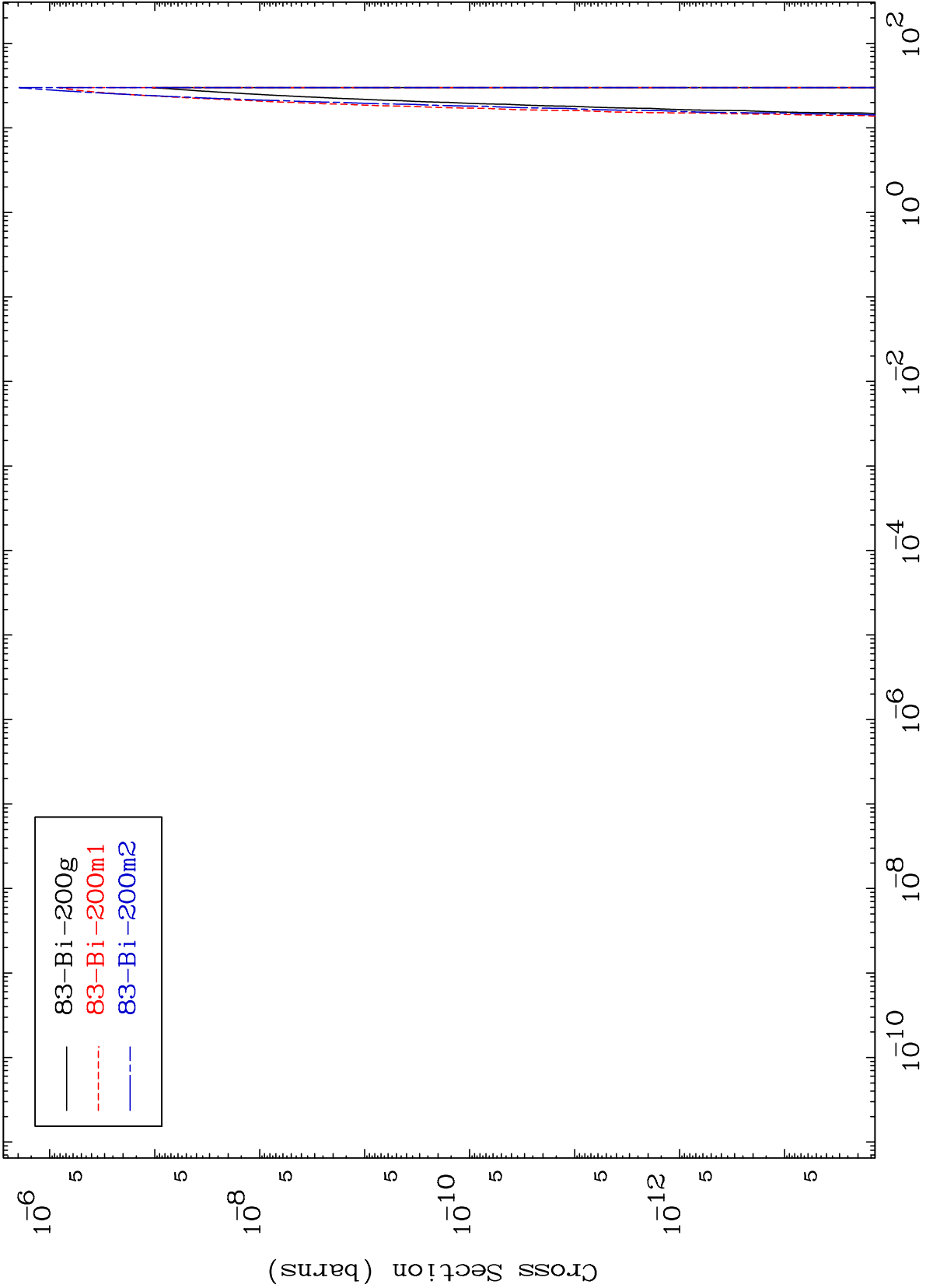
87-Fr-209

MAT 8716

(n,2n) 2 α

87-Fr-209

Radionuclide Production Cross Section



43

Incident Energy (MeV)

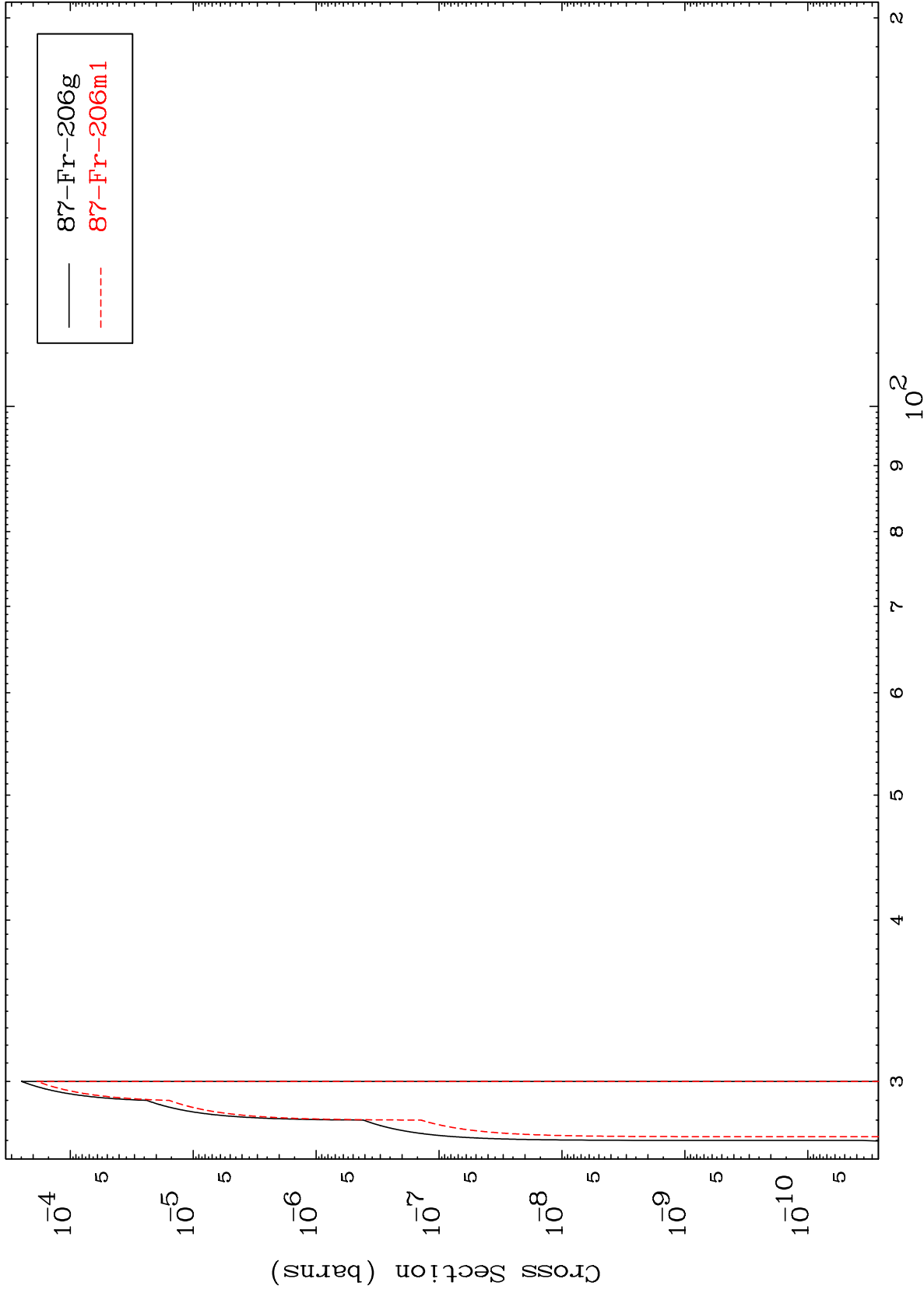
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MAT 8716

(n,4n)

87-Fr-209

Radionuclide Production Cross Section



44

Incident Energy (MeV)

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