

Program EVALPLOT
(Version 2021-1)

by

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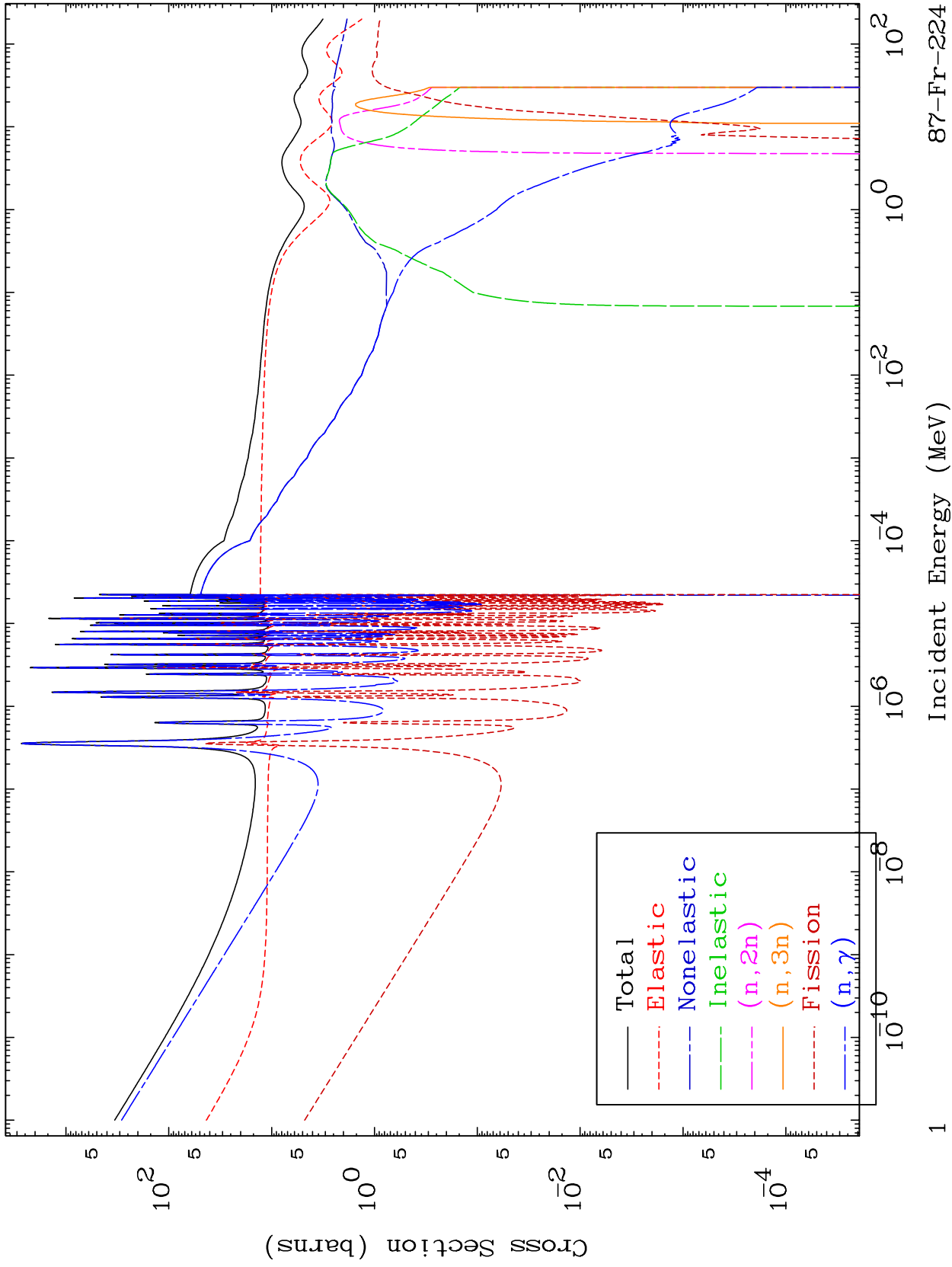
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Press Mouse Button to Start

MAT 8761

Neutron Major
293 Kelvin Cross Sections

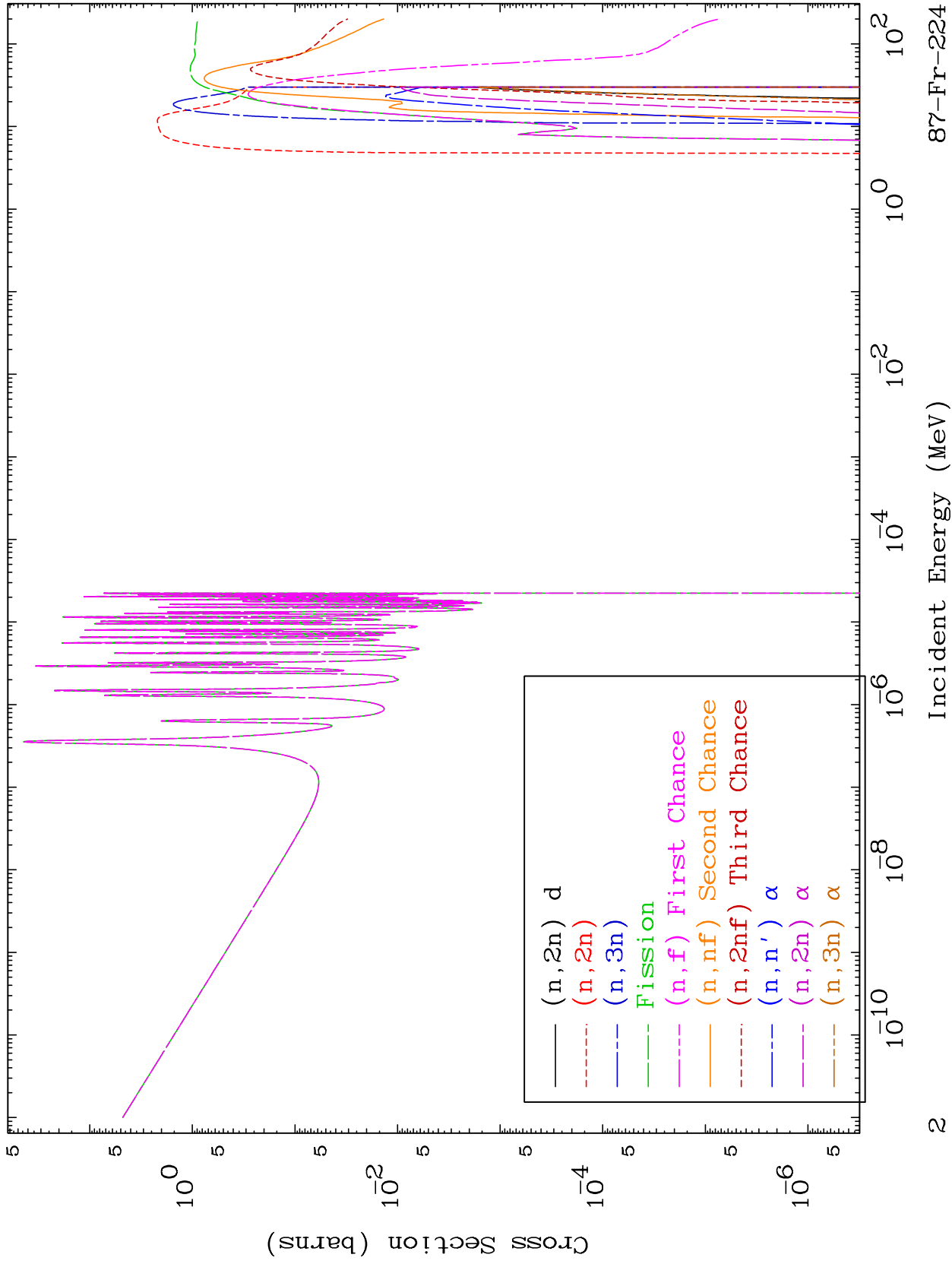
87-Fr-224



MAT 8761

Neutron Absorption
293 Kelvin Cross Sections

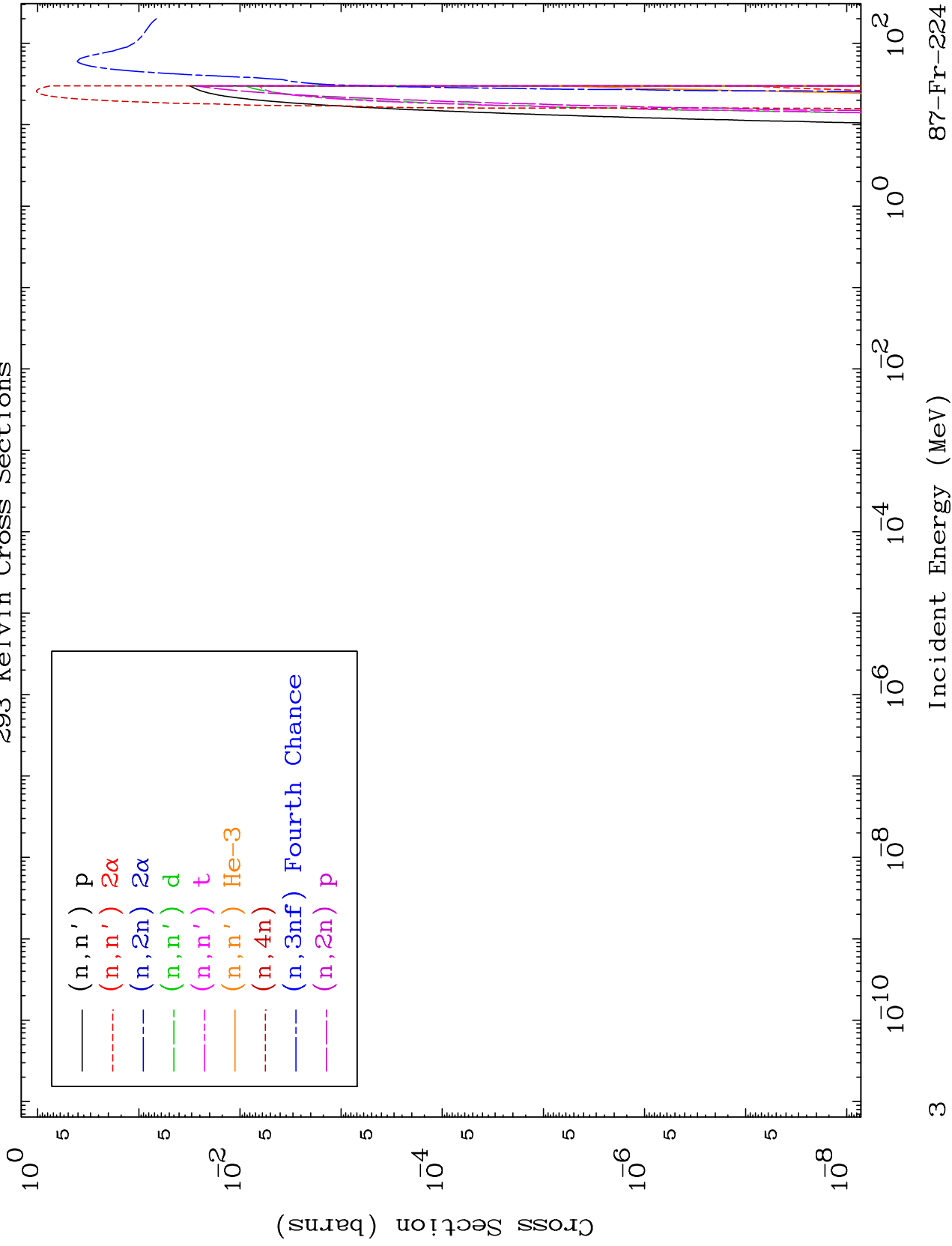
87-Fr-224



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Neutron Absorption
293 Kelvin Cross Sections

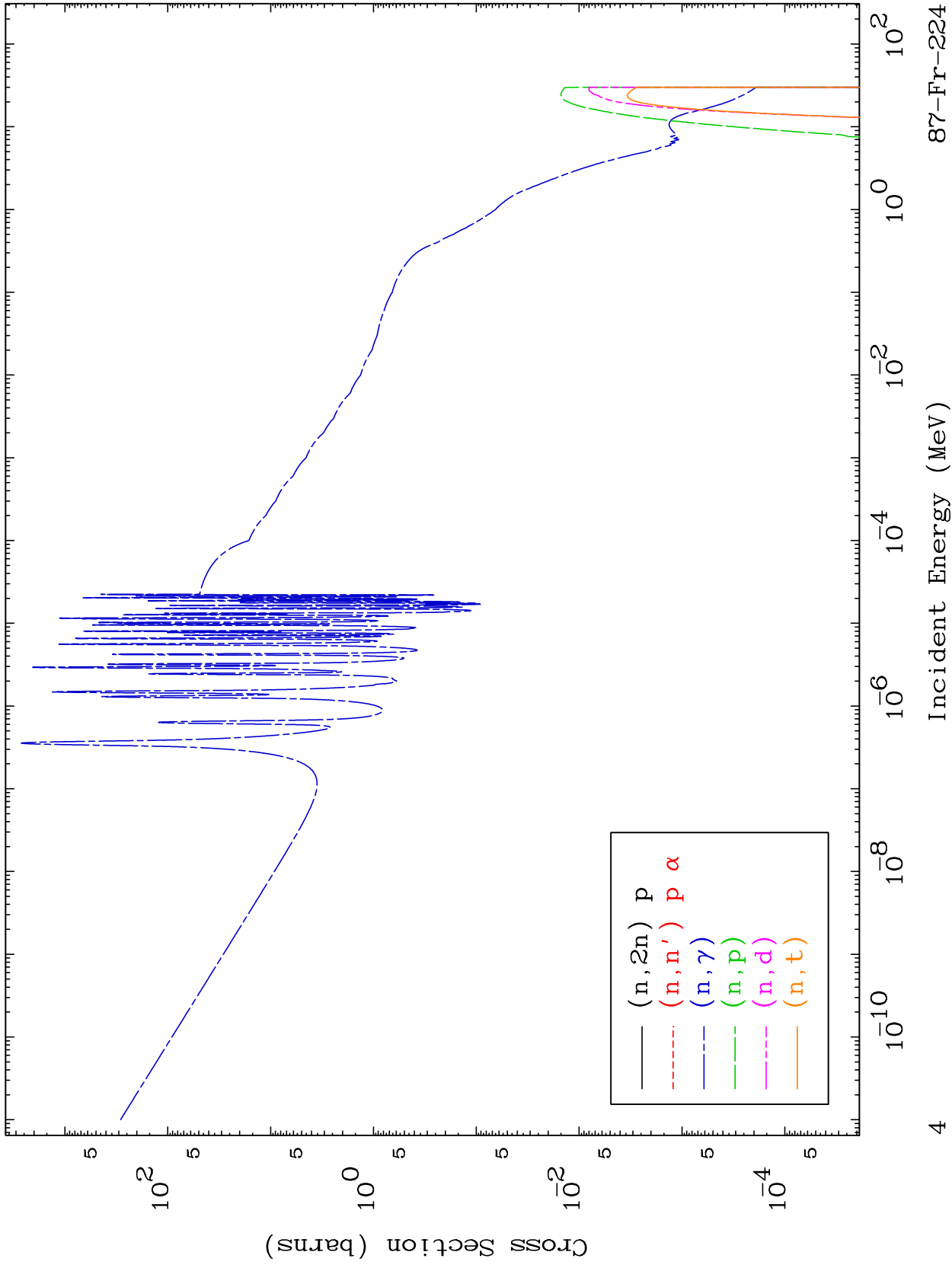
87-Fr-224



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Neutron Absorption
293 Kelvin Cross Sections

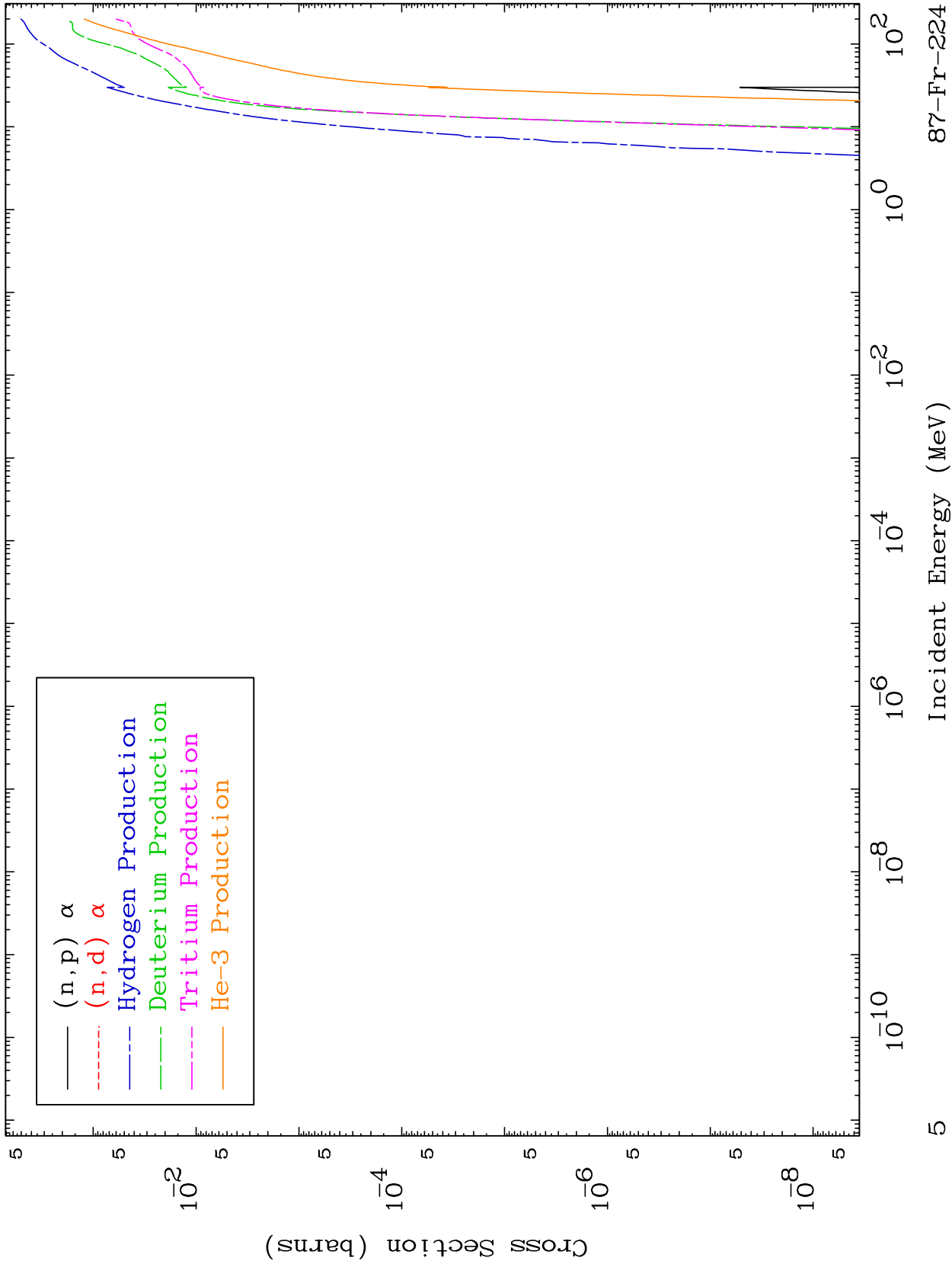
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Neutron Absorption
293 Kelvin Cross Sections

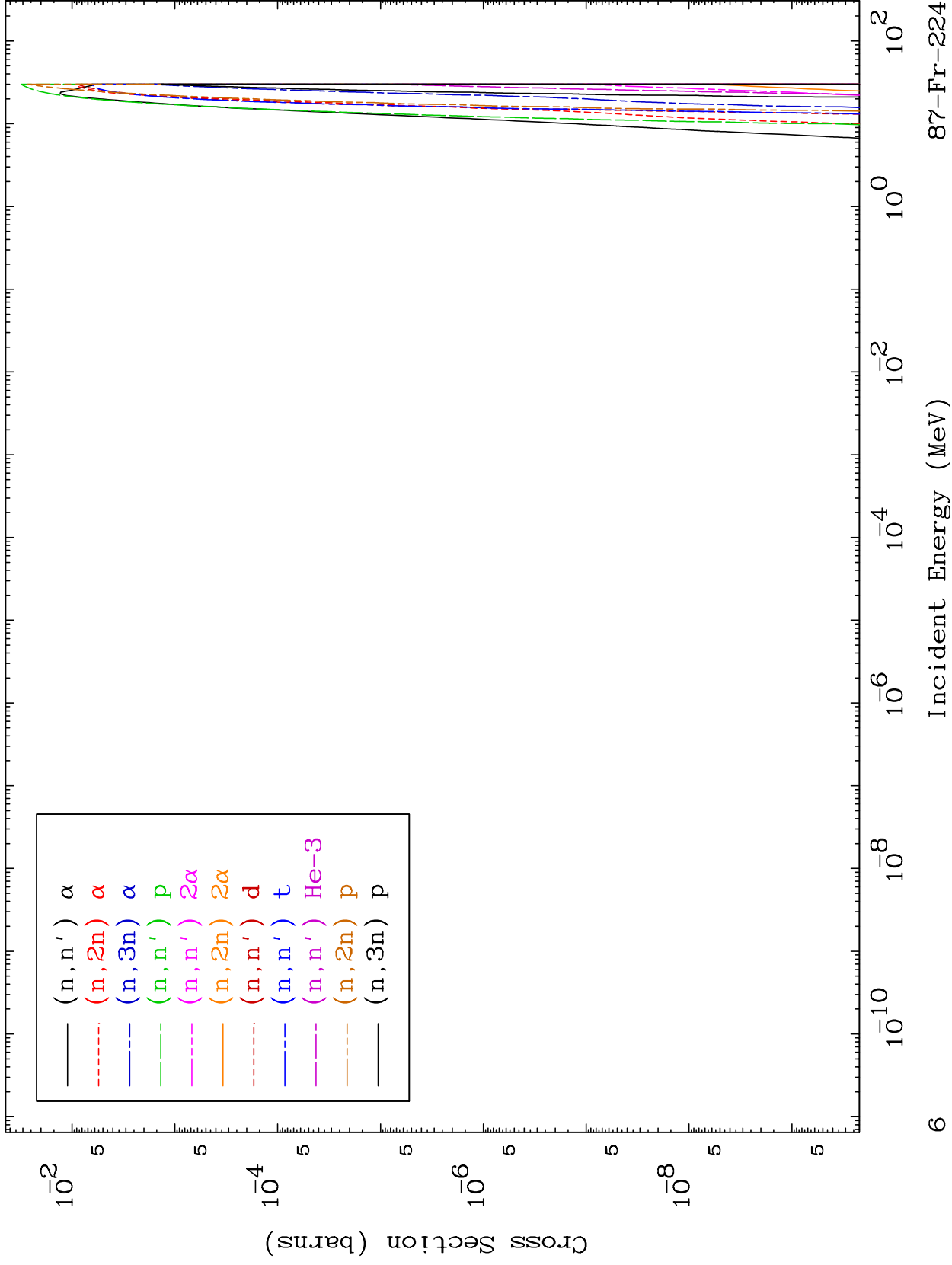
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Charged Particle
293 Kelvin Cross Sections

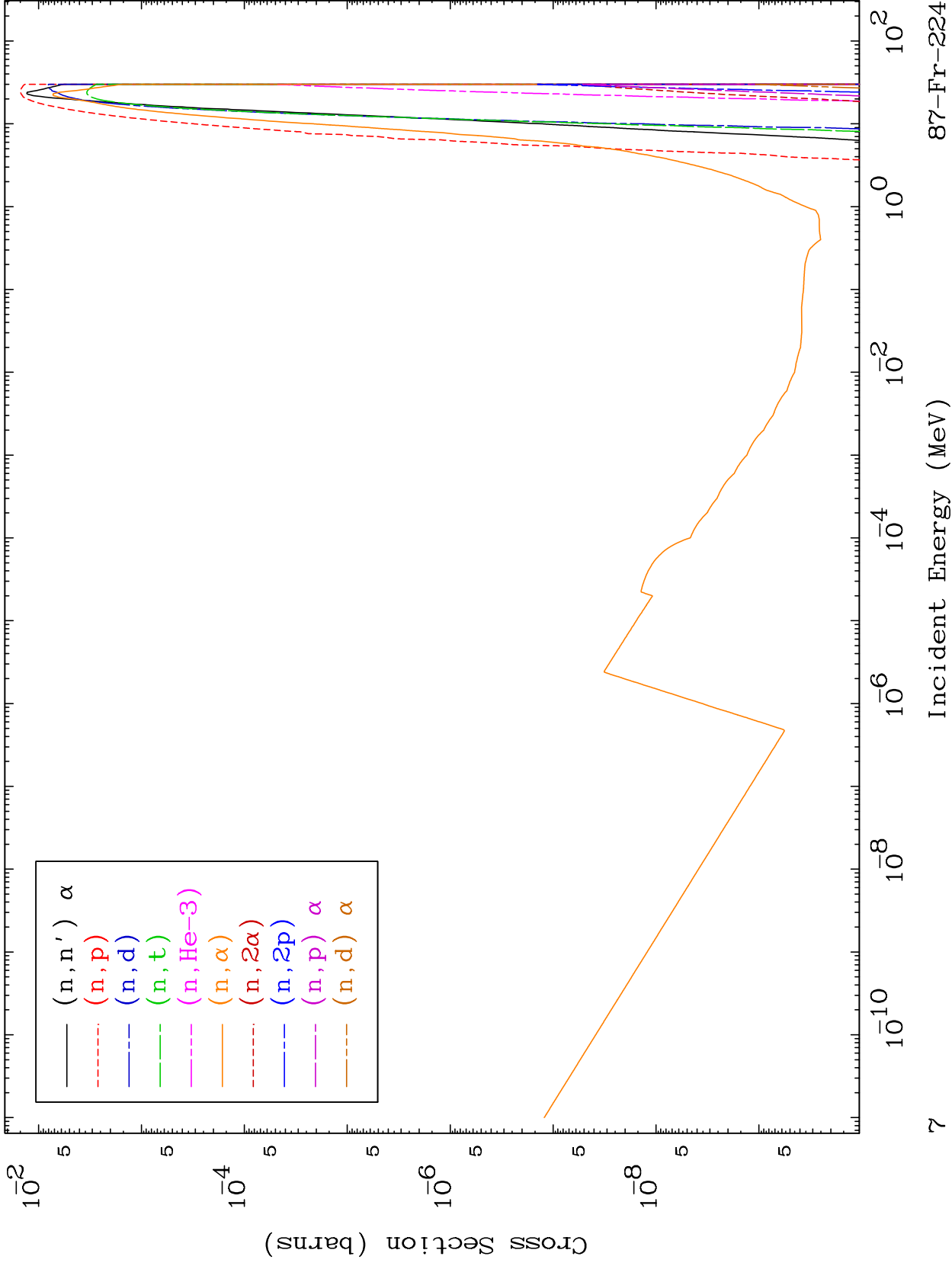
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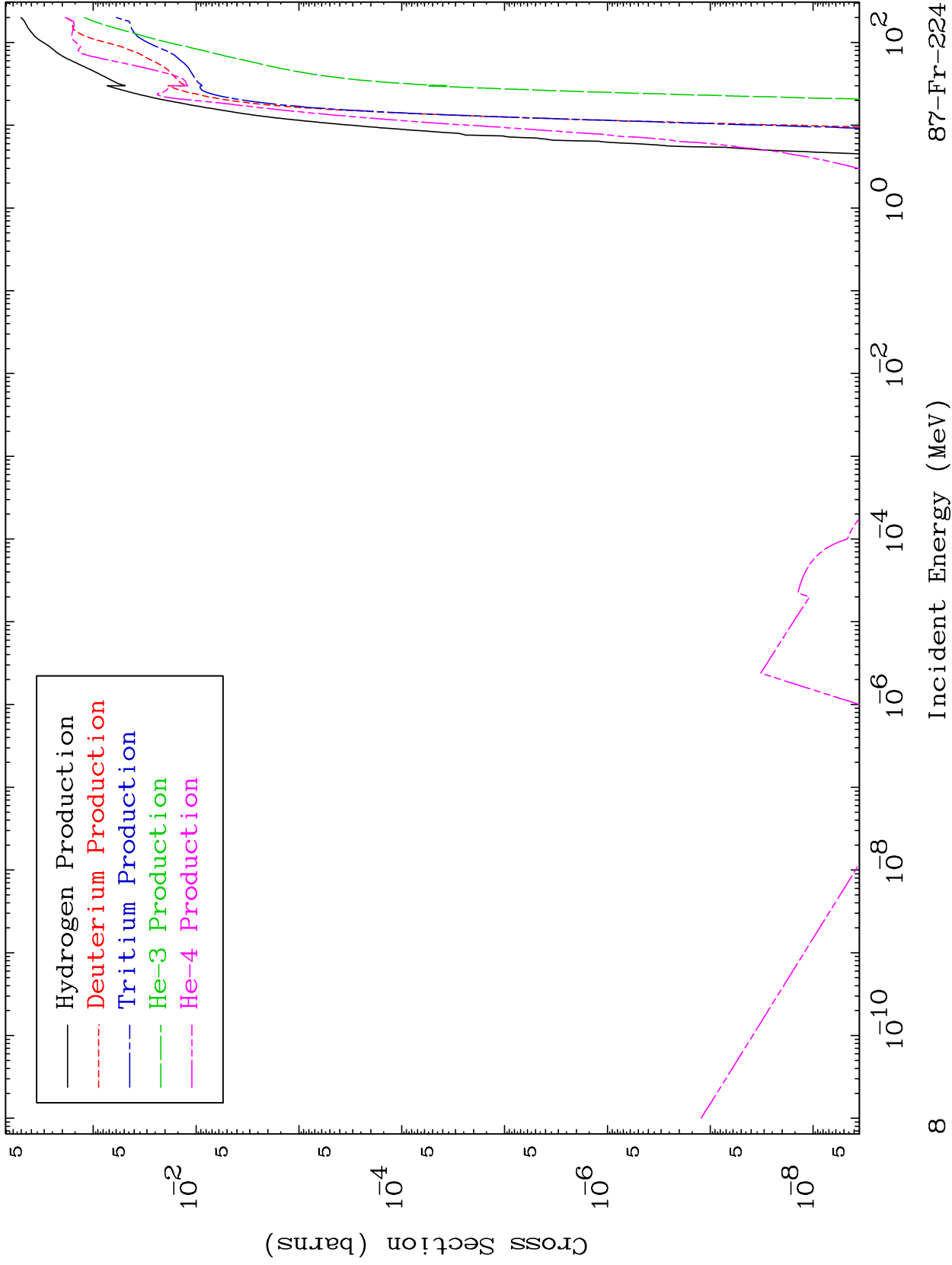


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Charged Particle
293 Kelvin Cross Sections

87-Fr-224

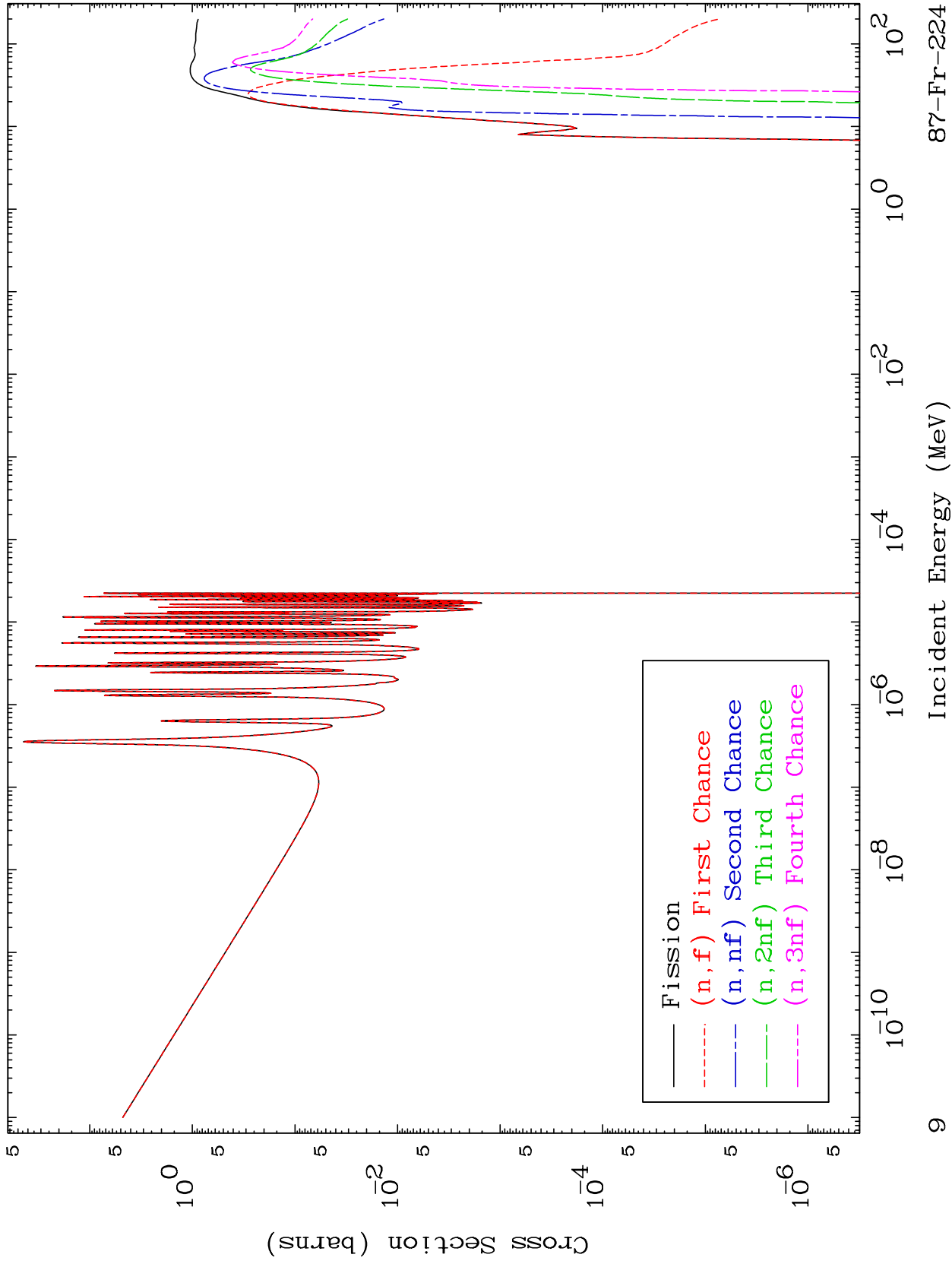




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Fission
293 Kelvin Cross Sections

87-Fr-224

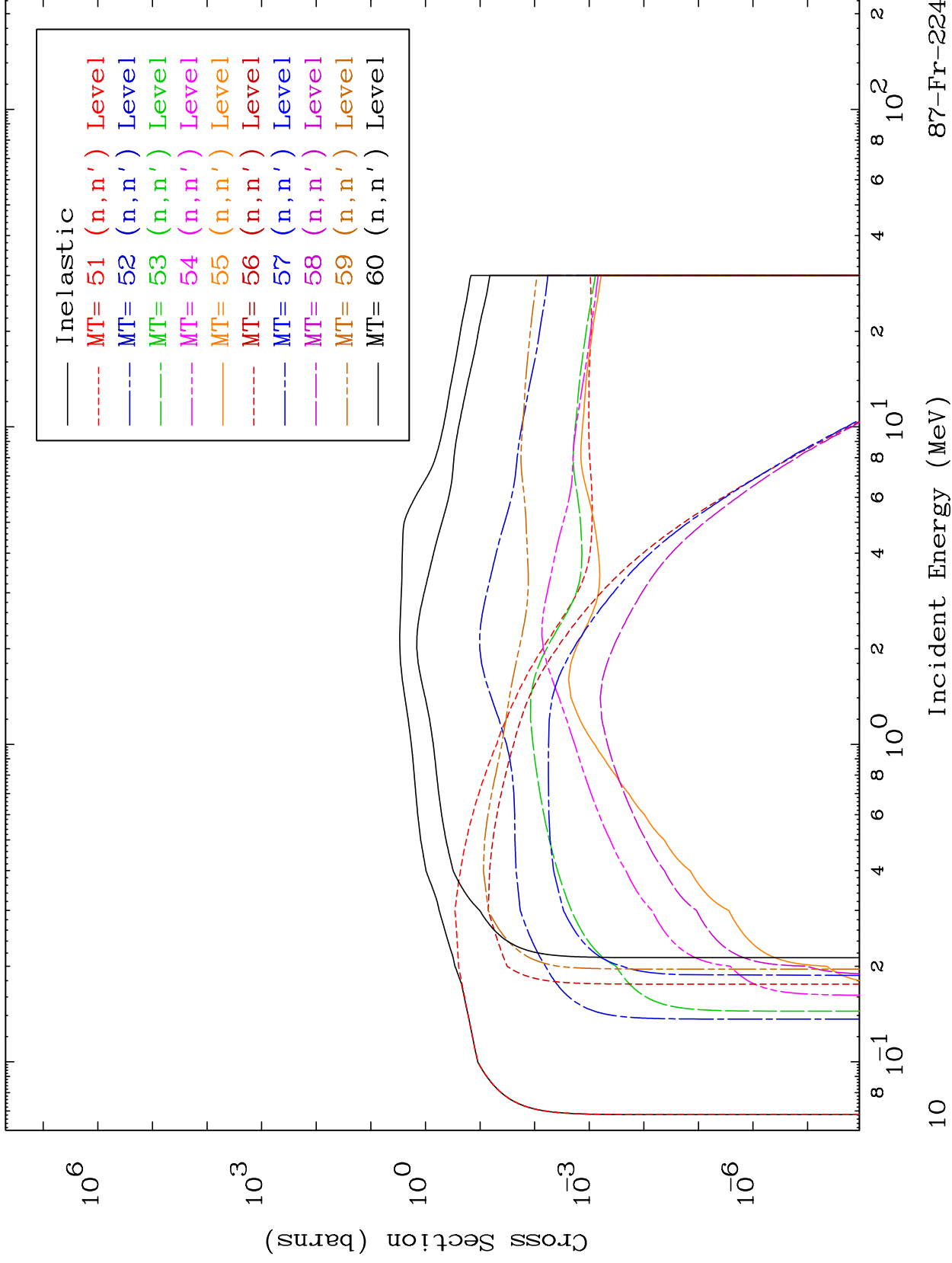


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(n,n') Levels

87-Fr-224

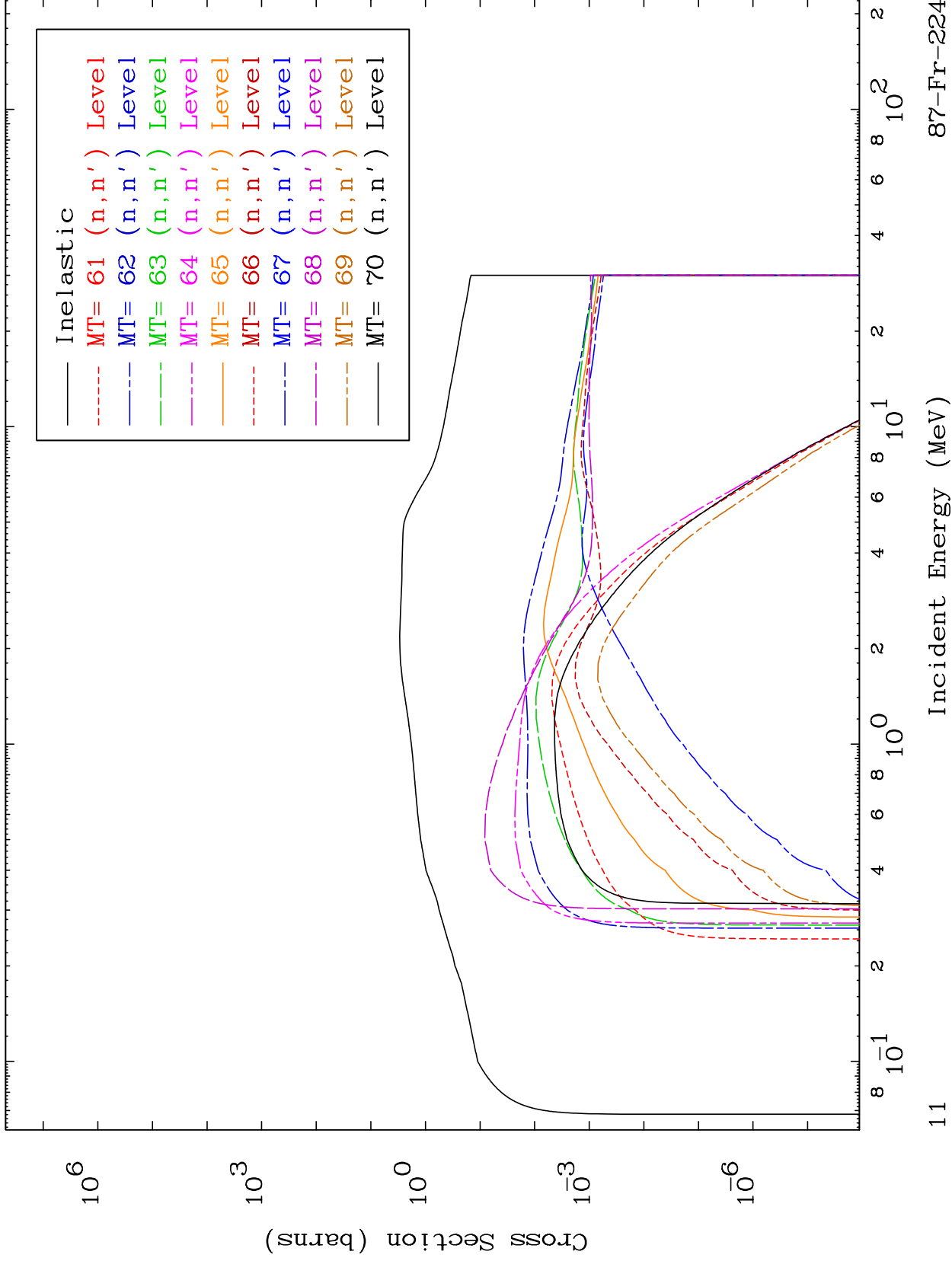
293 Kelvin Cross Sections



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Incident Energy (MeV)

87-Fr-224

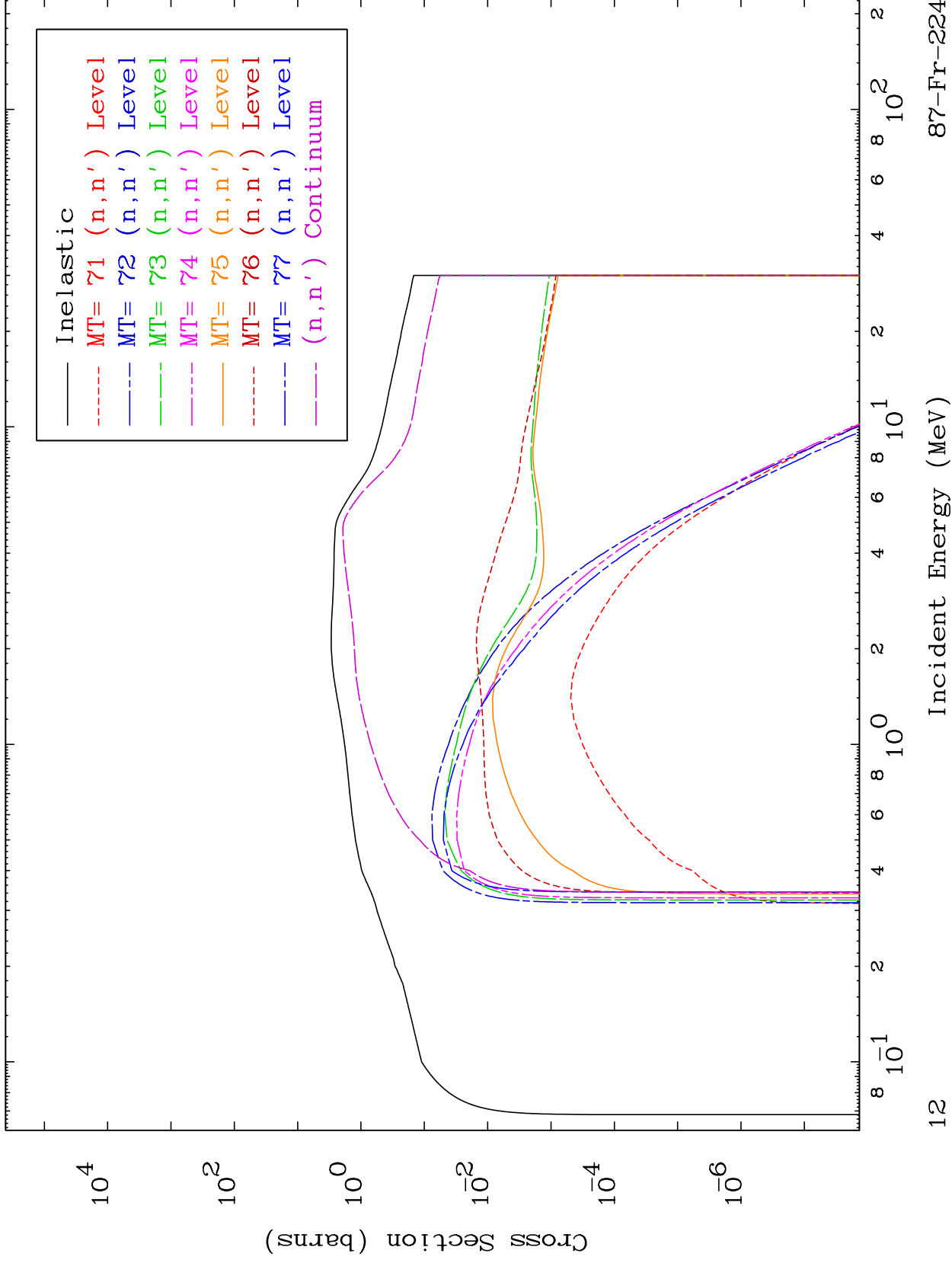


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(n,n') Levels

293 Kelvin Cross Sections

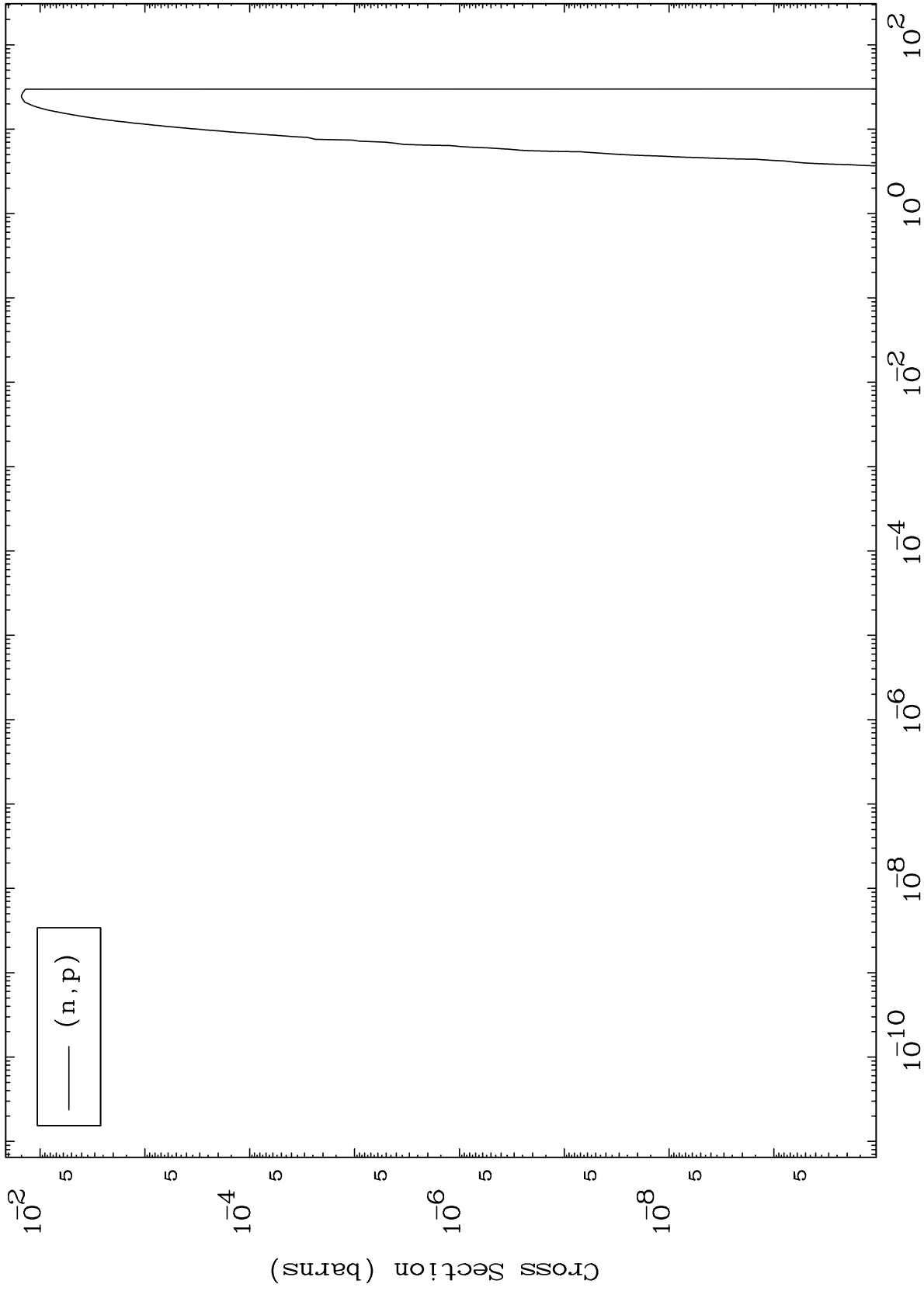
87-Fr-224



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(n,p) Levels
293 Kelvin Cross Sections

87-Fr-224



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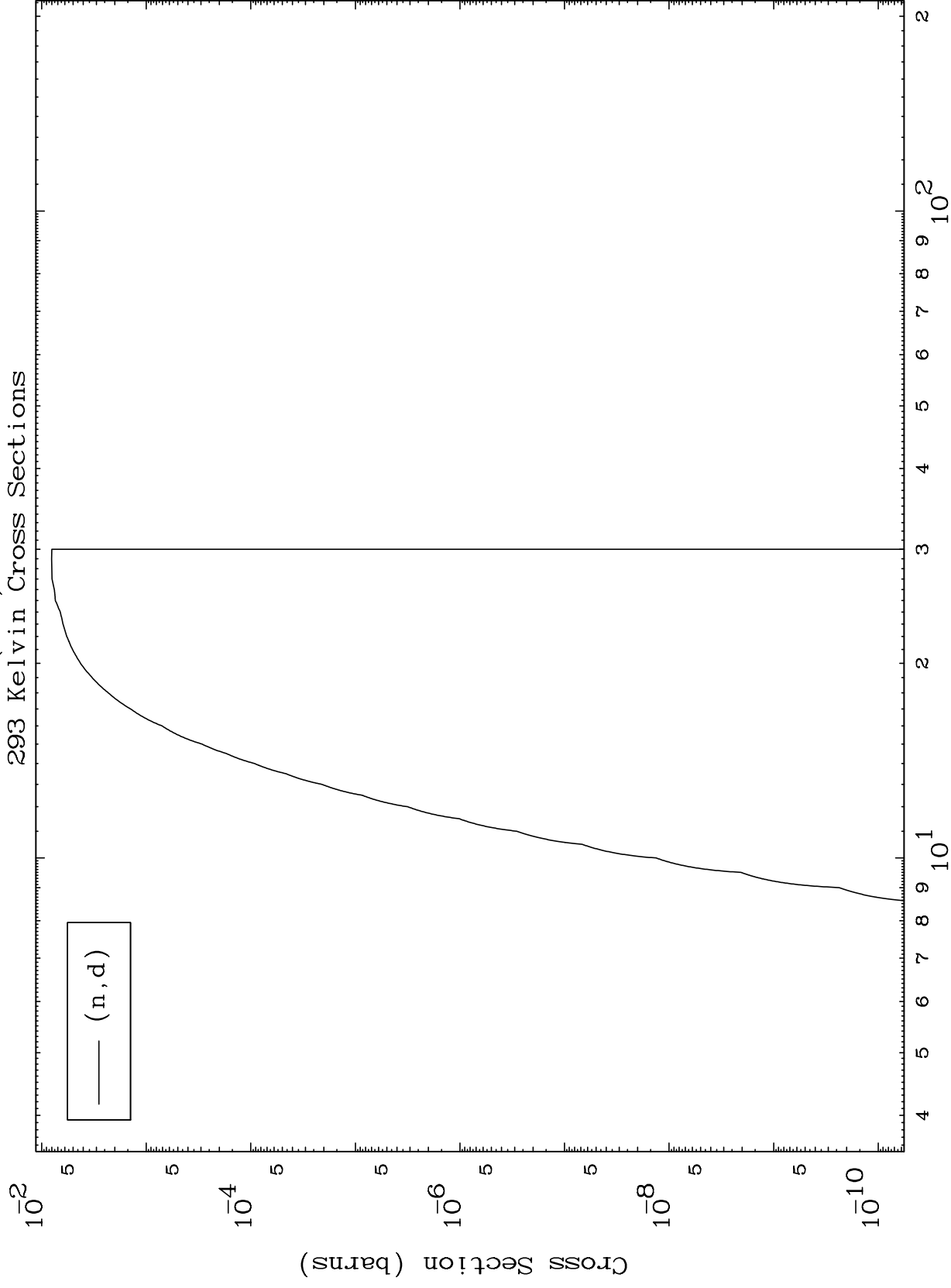
Incident Energy (MeV)

87-Fr-224

MAT 8761

(n,d) Levels
293 Kelvin Cross Sections

87-Fr-224



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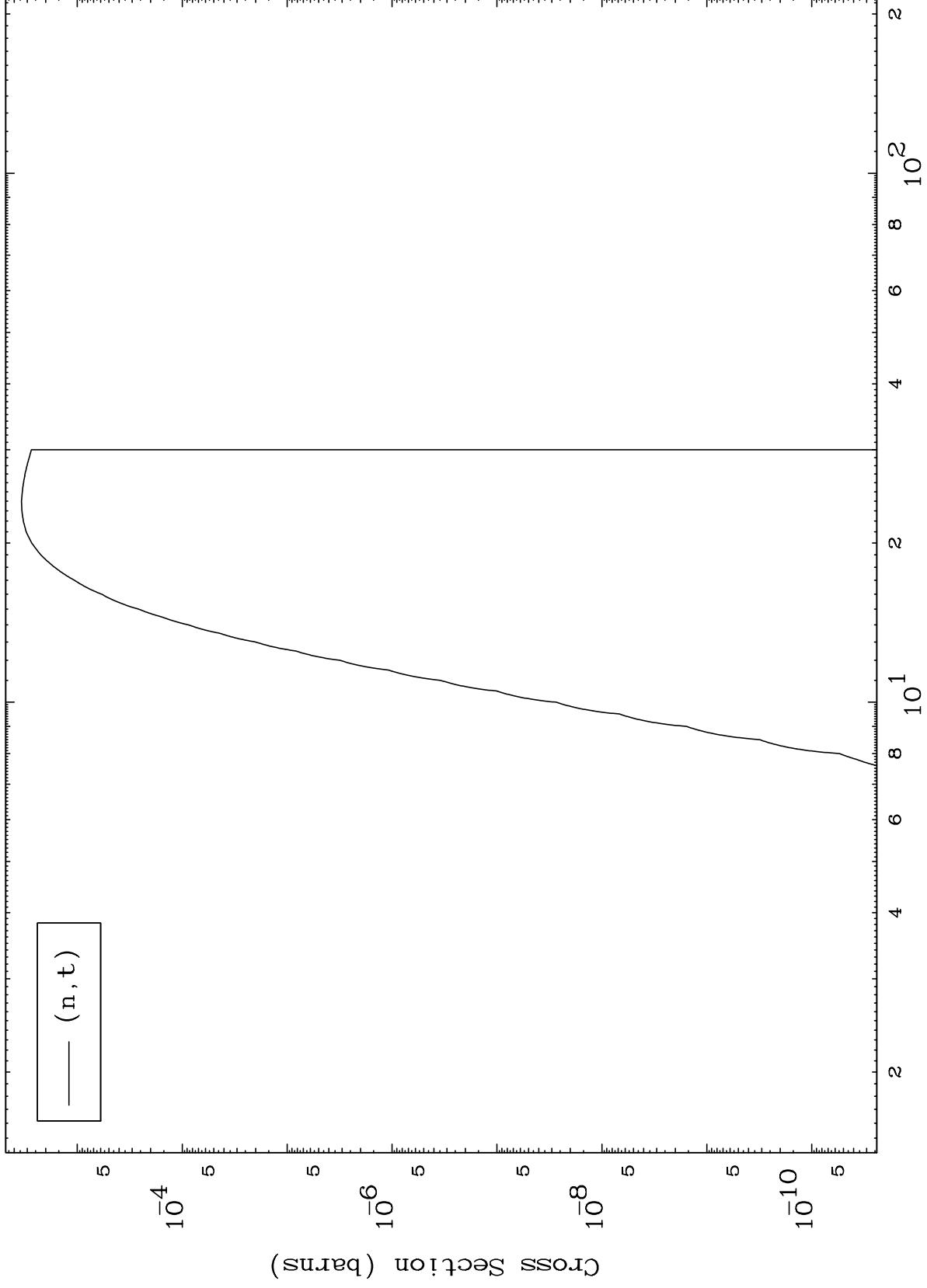
Incident Energy (MeV)

87-Fr-224

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(n,t) Levels
293 Kelvin Cross Sections

87-Fr-224



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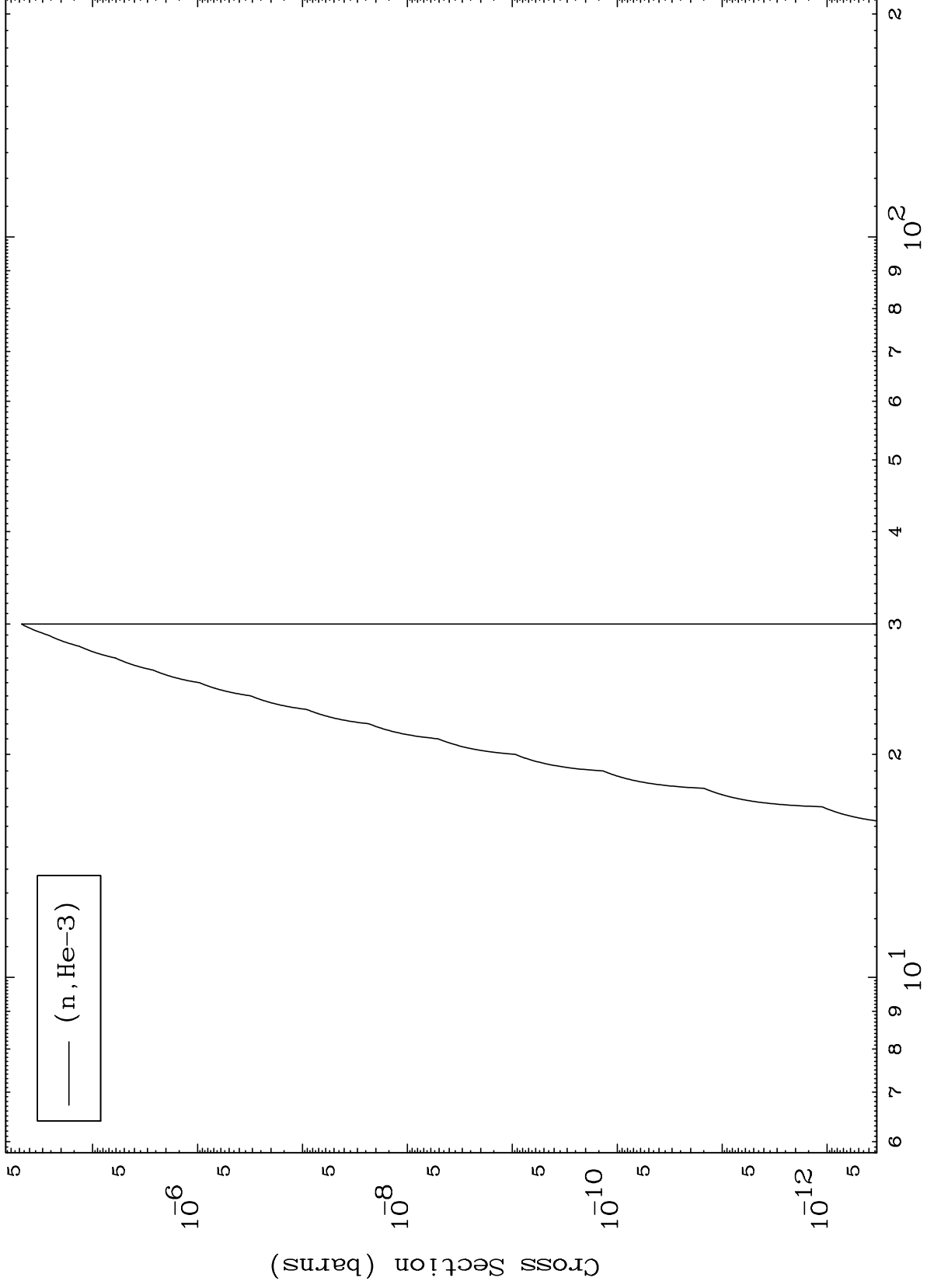
Incident Energy (MeV)

87-Fr-224

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(n,He3) Levels
293 Kelvin Cross Sections

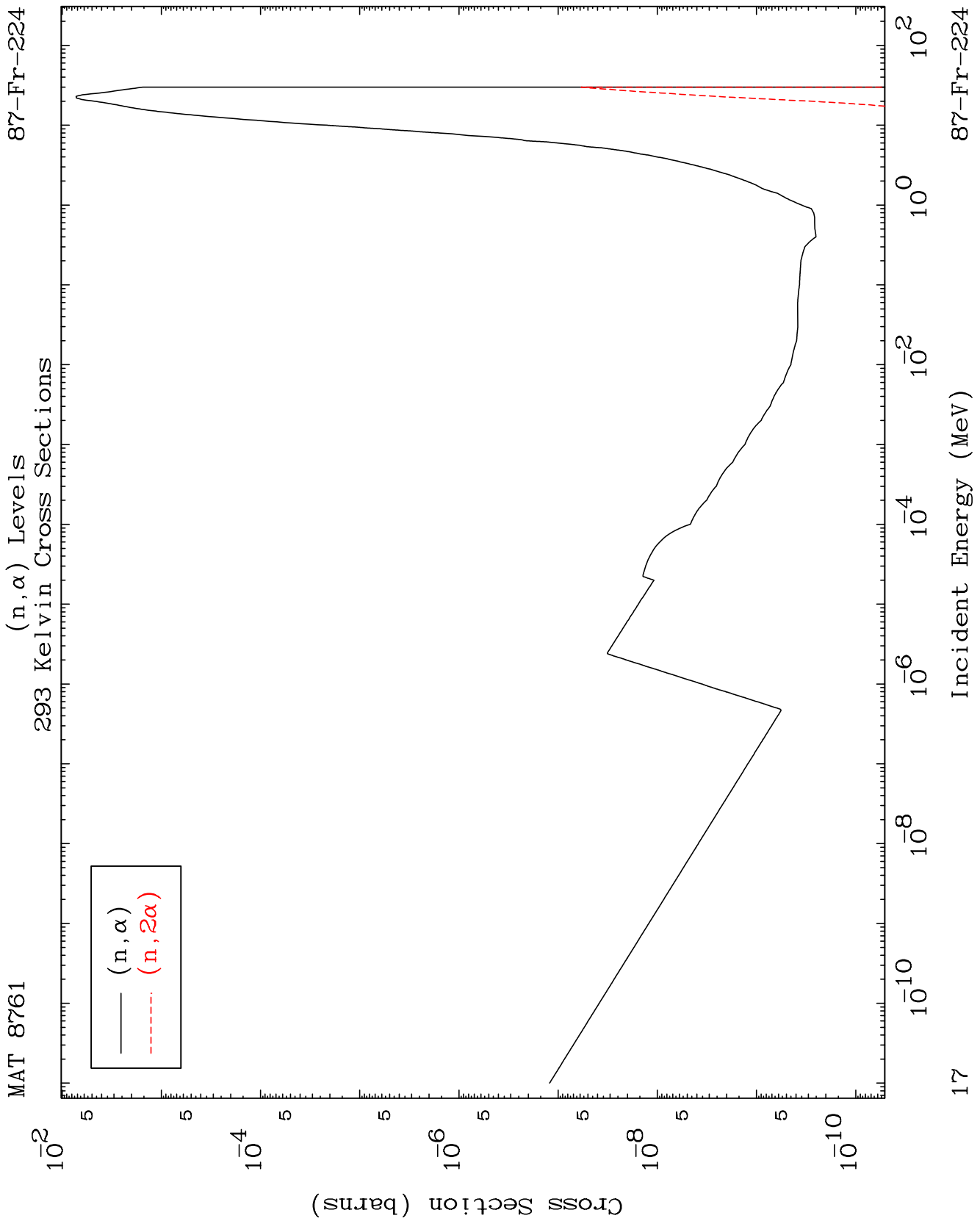
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16

Incident Energy (MeV)

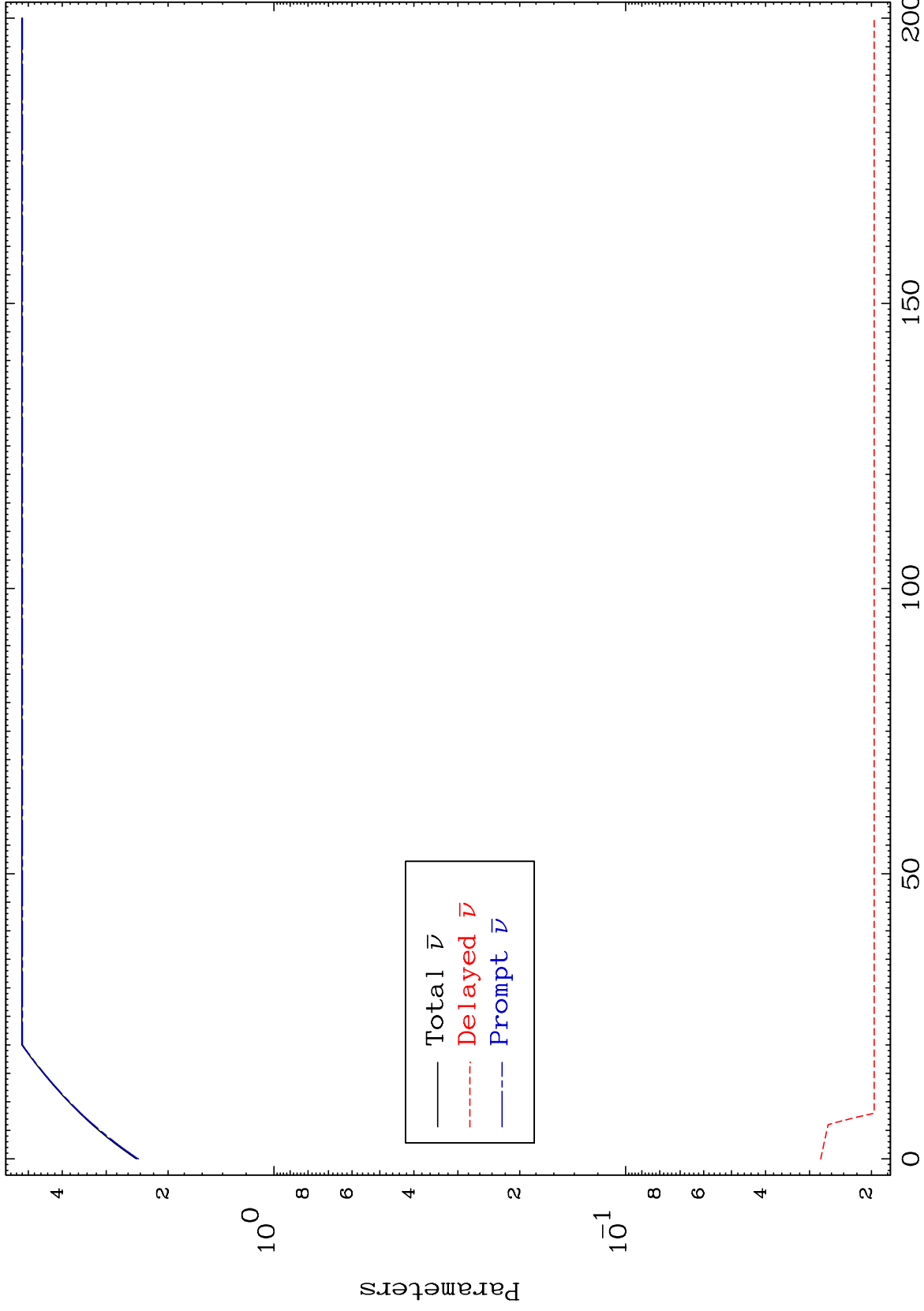
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Energy Release
Parameters

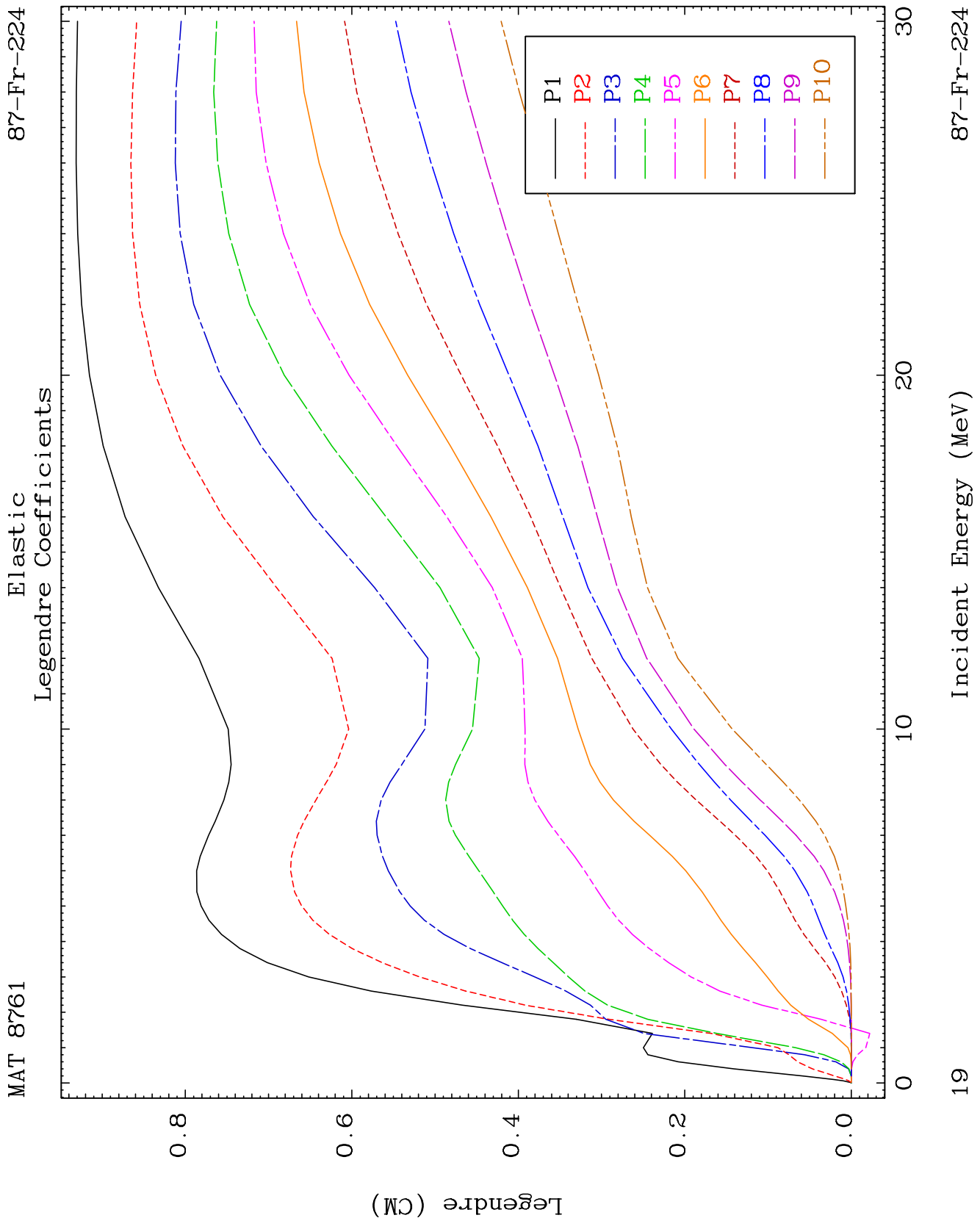
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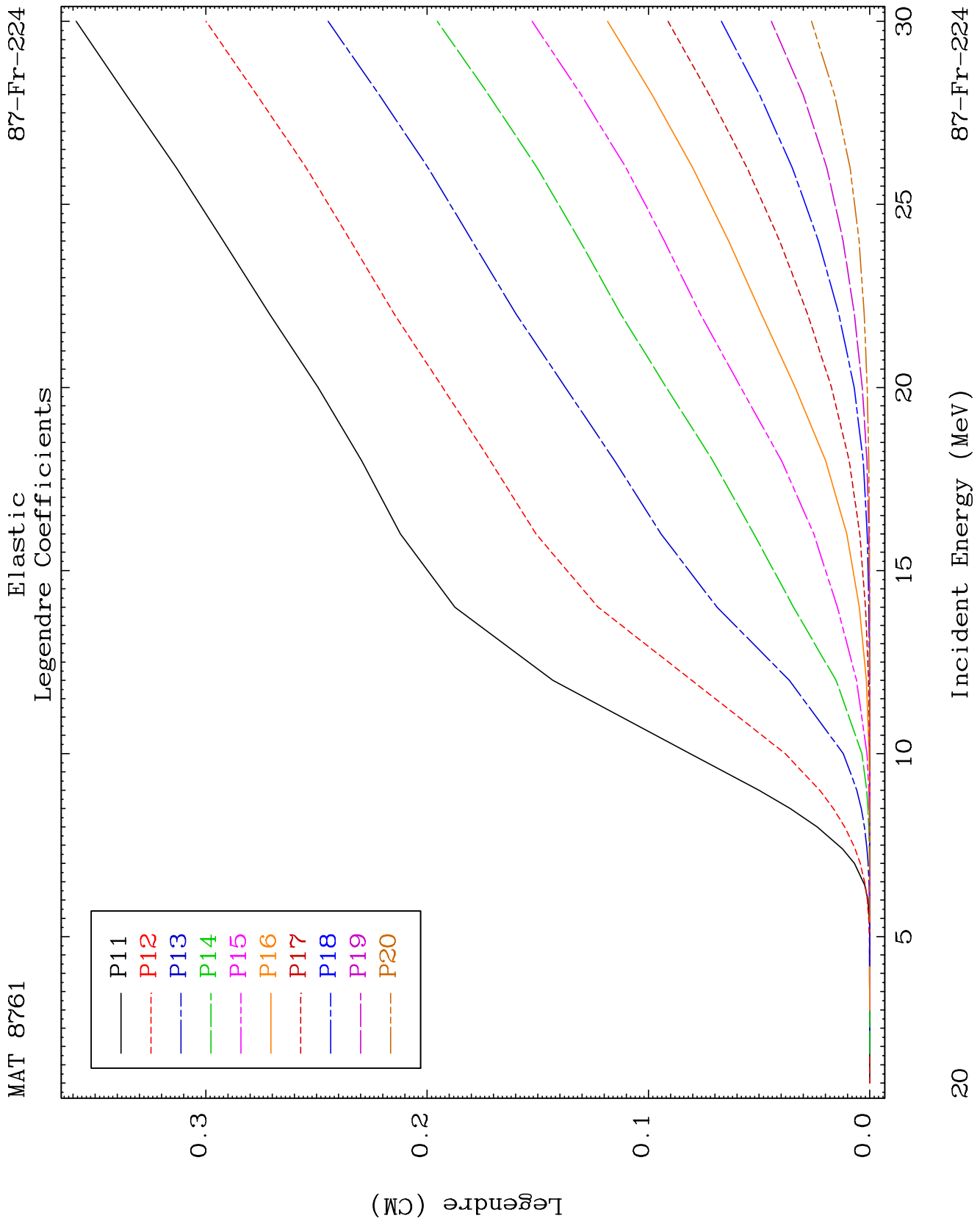


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Incident Energy (MeV)

87-Fr-224

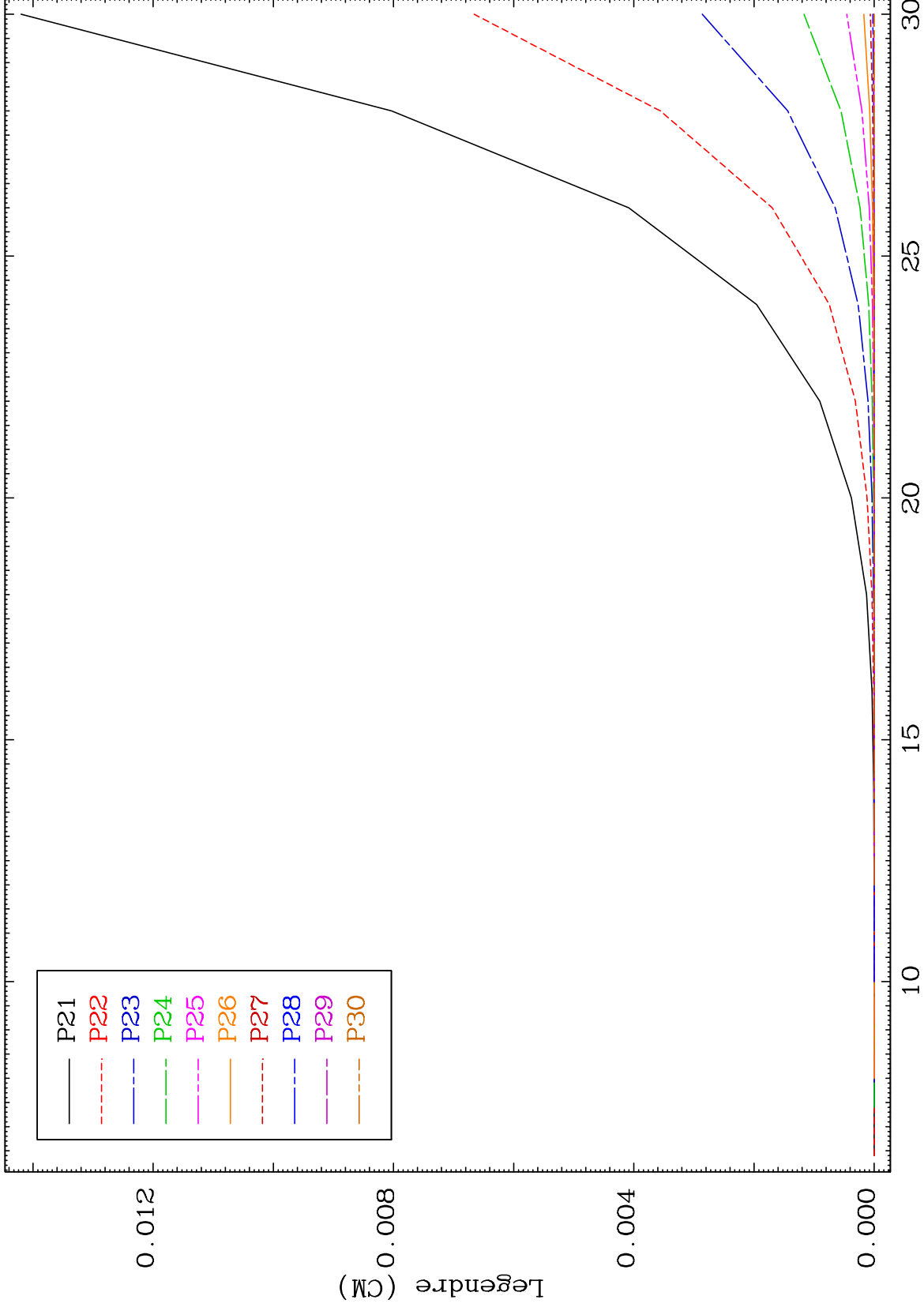




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Elastic
Legendre Coefficients

87-Fr-224



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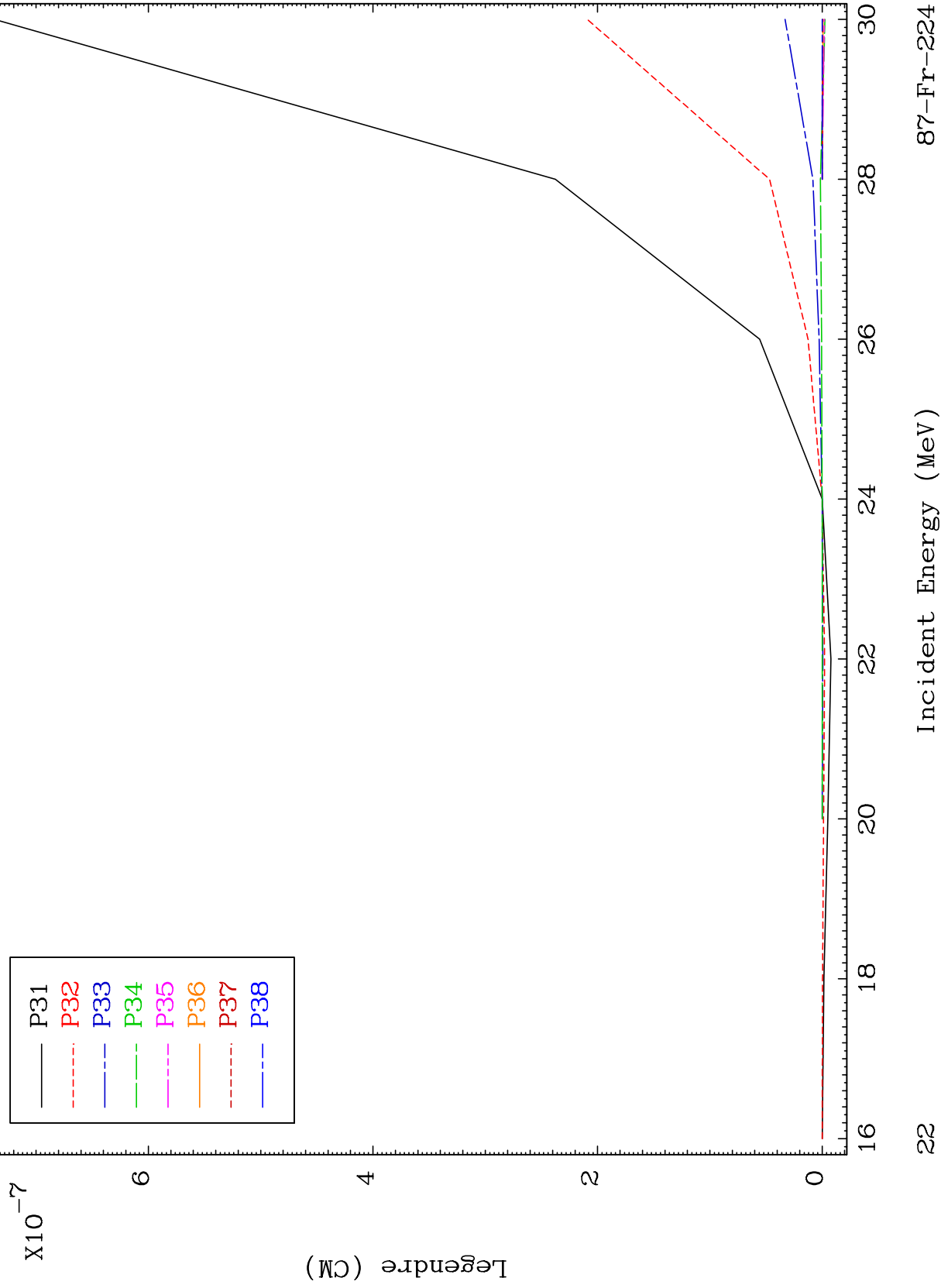
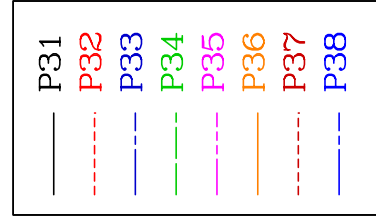
Incident Energy (MeV)

87-Fr-224

MAT 8761

Elastic
Legendre Coefficients

87-Fr-224



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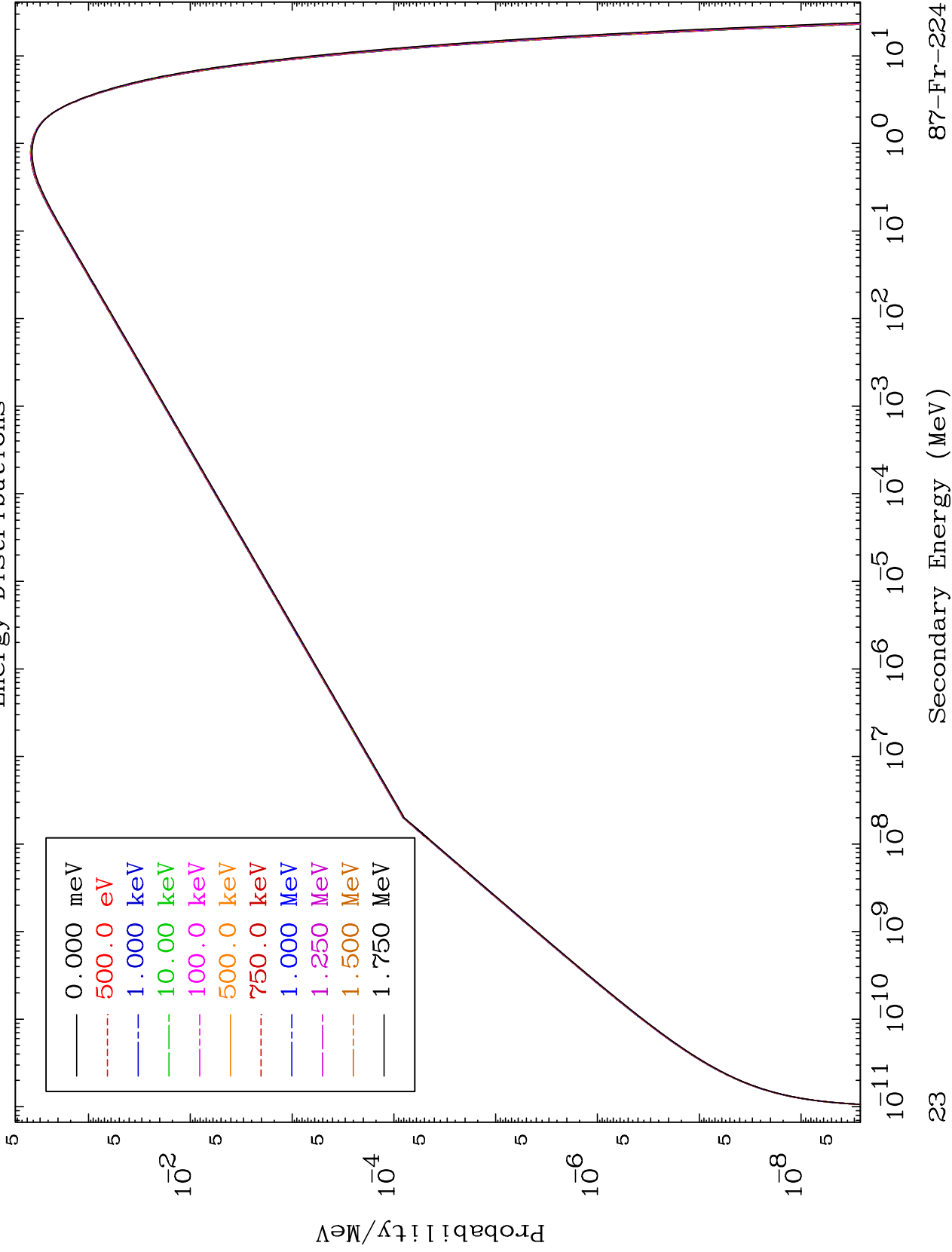
Incident Energy (MeV)

87-Fr-224

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Fission Energy Distributions

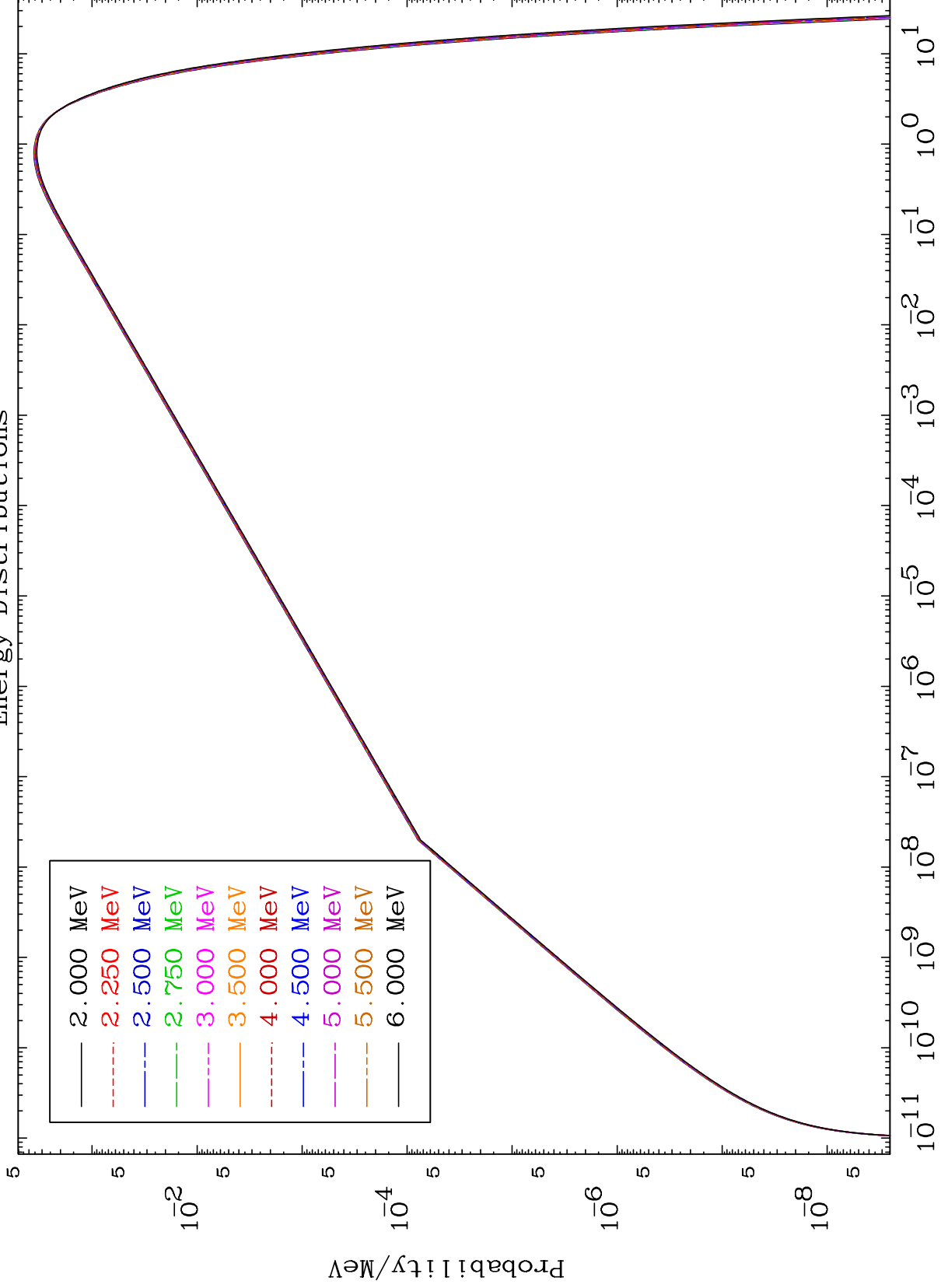
87-Fr-224



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Fission Energy Distributions

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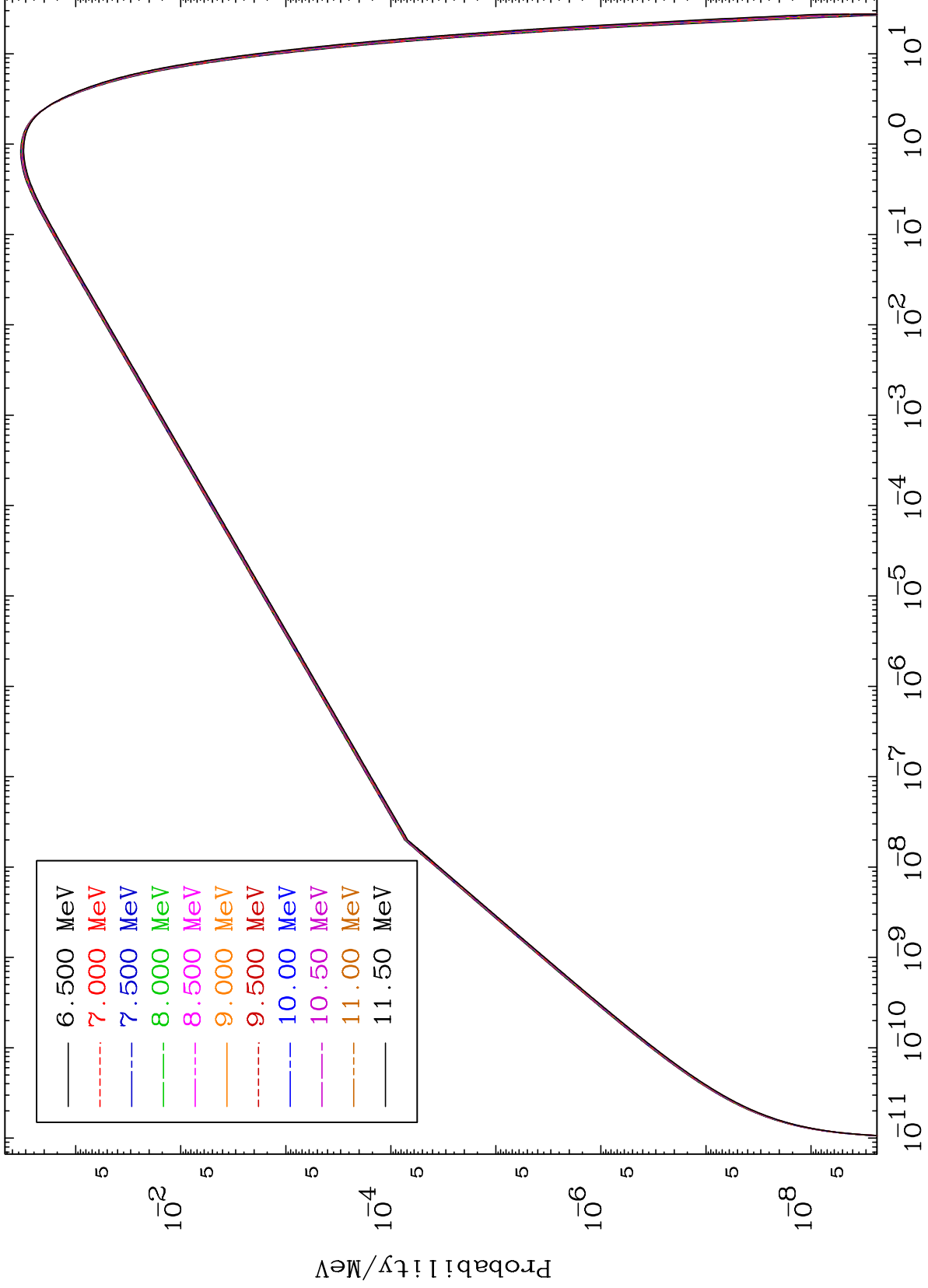
Secondary Energy (MeV)

87-Fr-224

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Fission Energy Distributions

87-Fr-224



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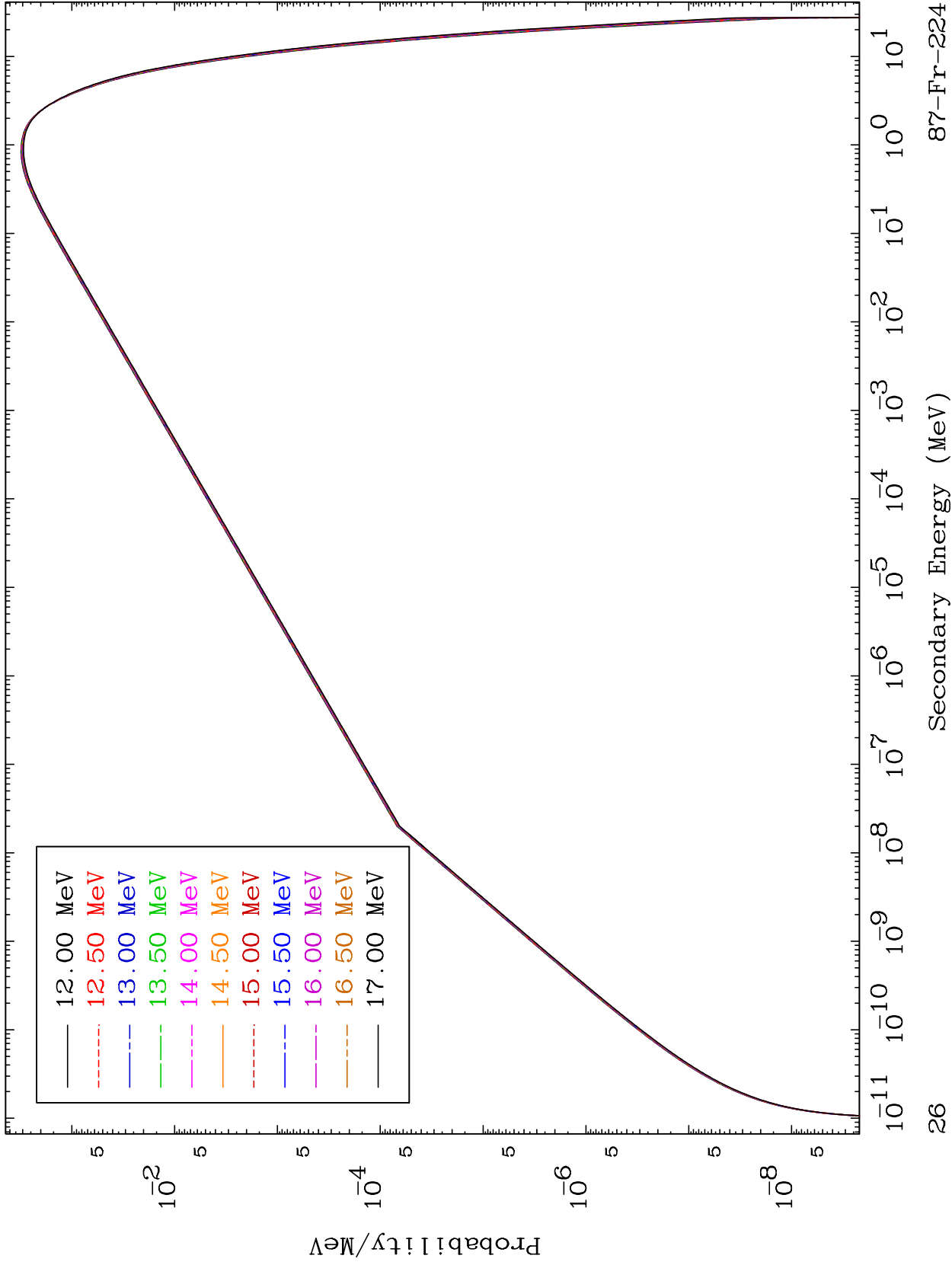
Secondary Energy (MeV)

87-Fr-224

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Fission Energy Distributions

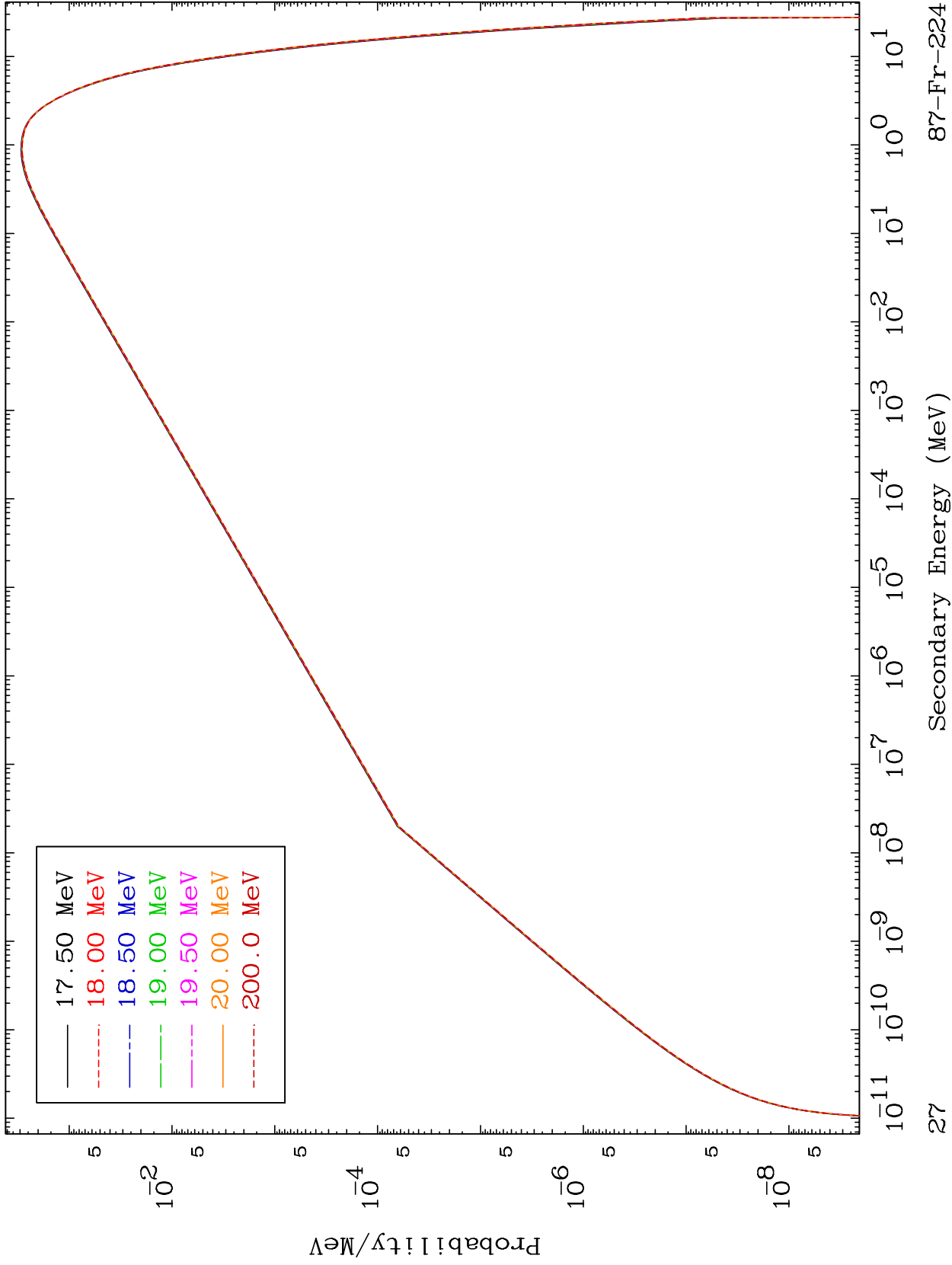
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Fission Energy Distributions

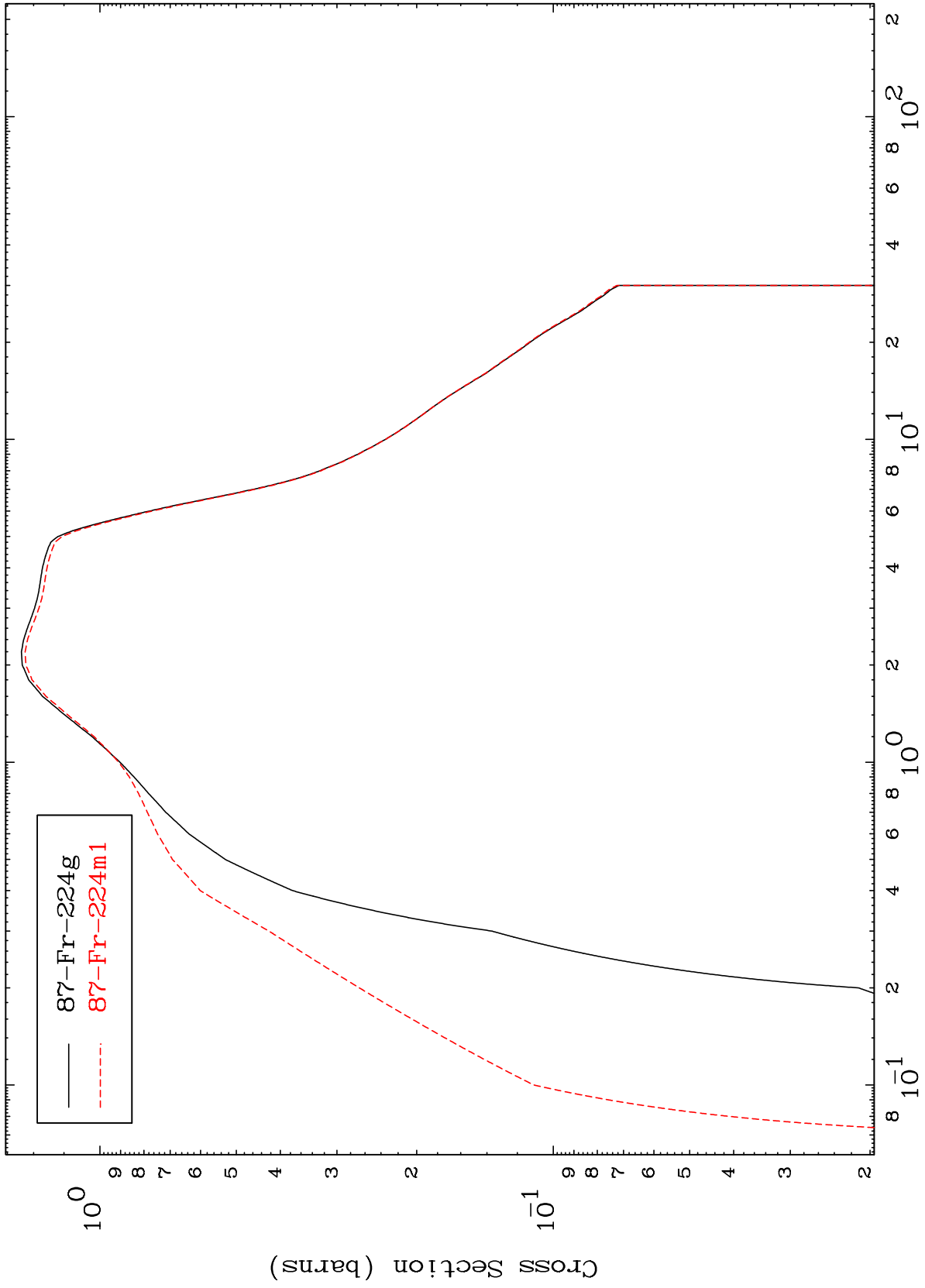
⁸⁷Fr-224



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Radionuclide Production Cross Section



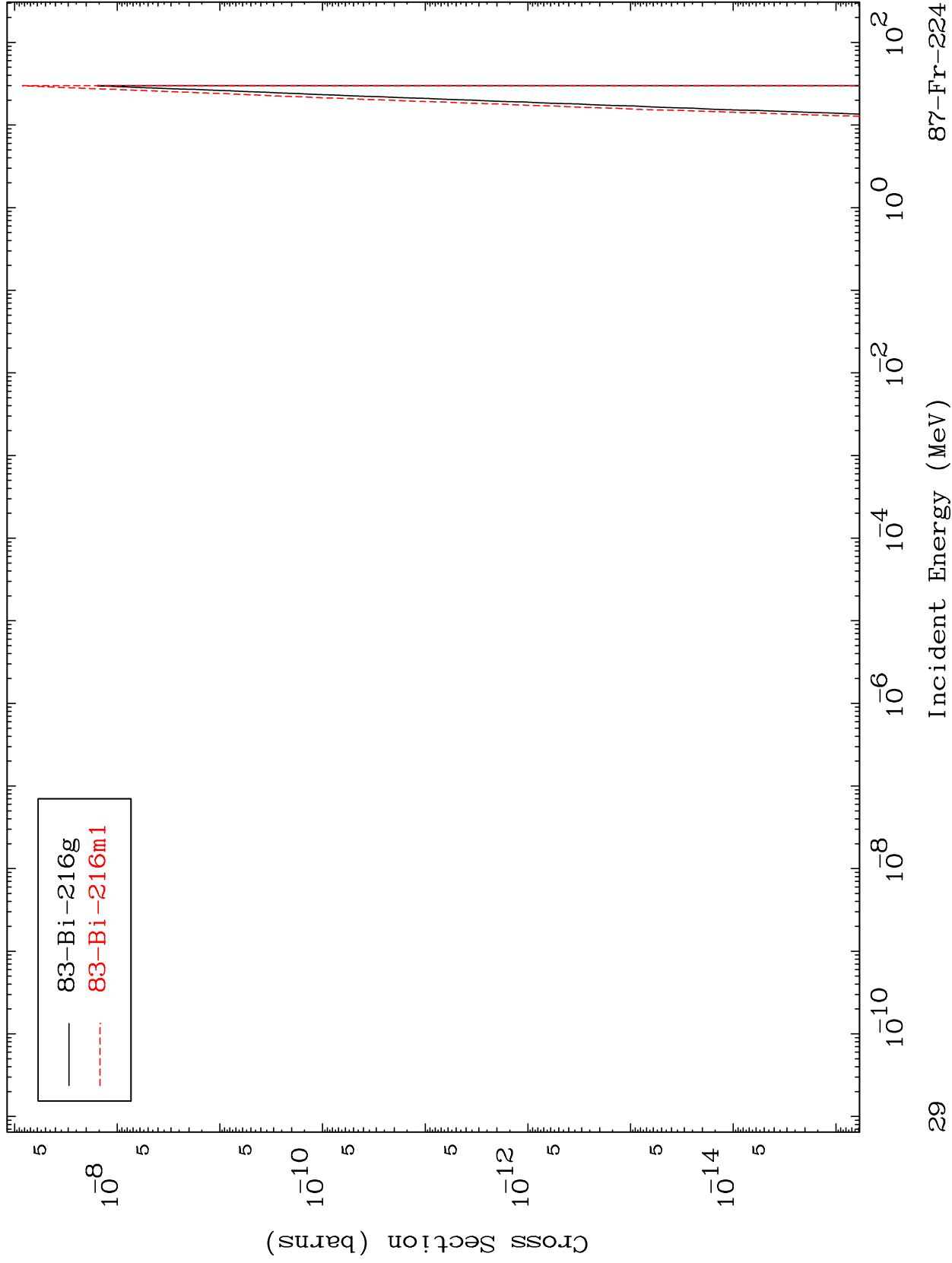
87-Fr-224g
87-Fr-224m1

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(n,n') 2α

87-Fr-224

Radionuclide Production Cross Section

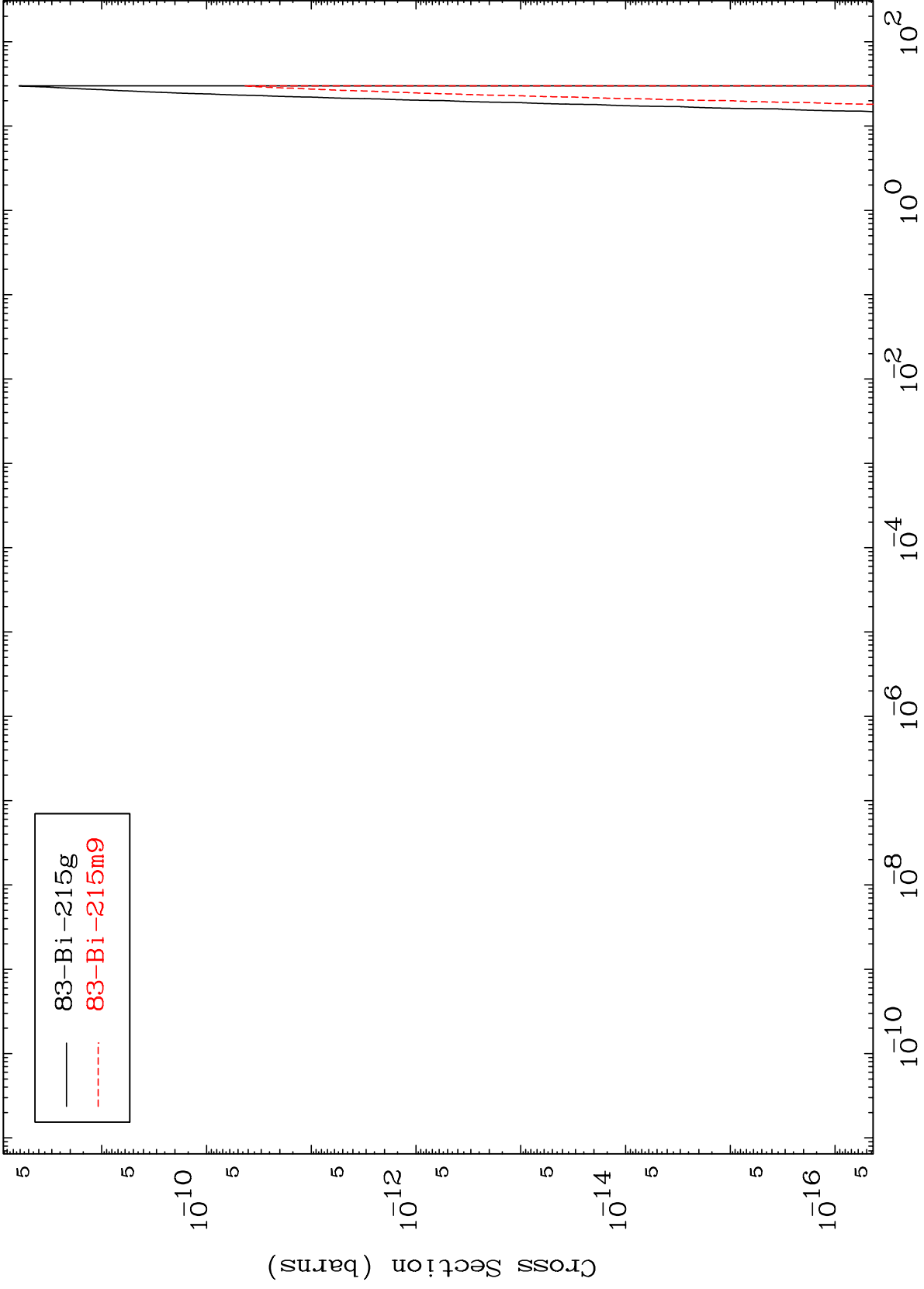


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(n,2n) 2 α

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Radionuclide Production Cross Section



30

Incident Energy (MeV)

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