

Program EVALPLOT
(Version 2021-1)

by

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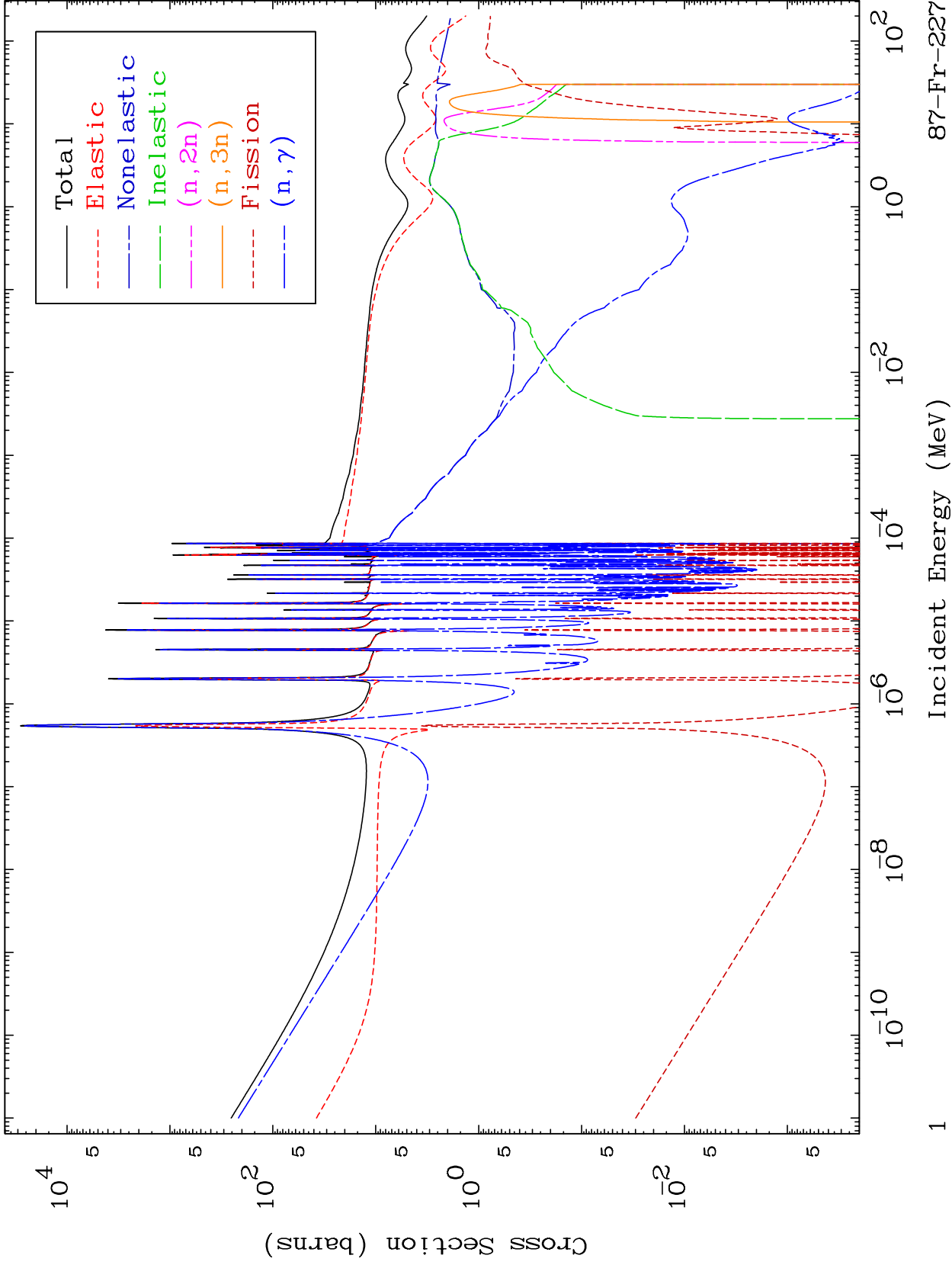
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Press Mouse Button to Start

MAT 8770

Neutron Major
293 Kelvin Cross Sections

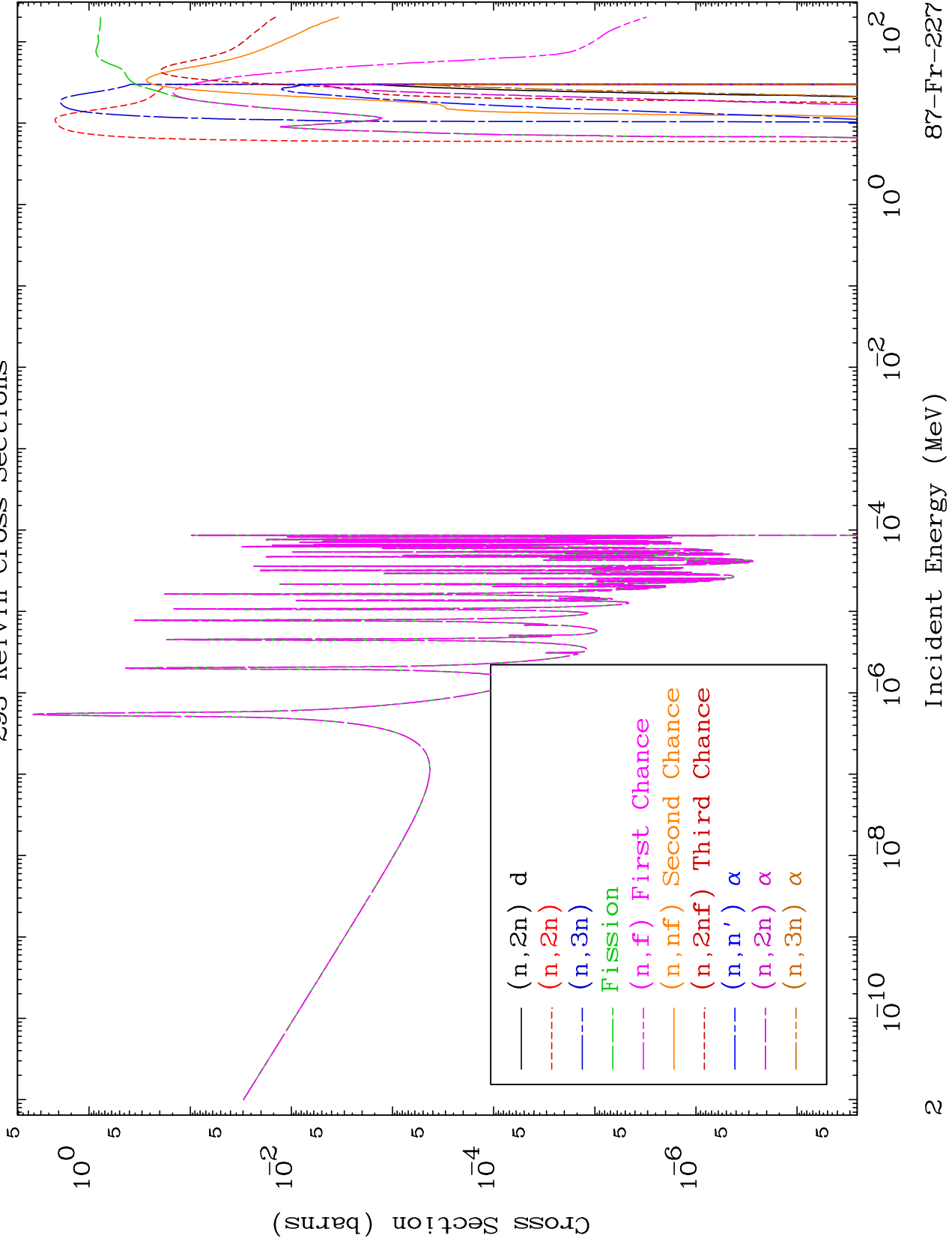
87-Fr-227

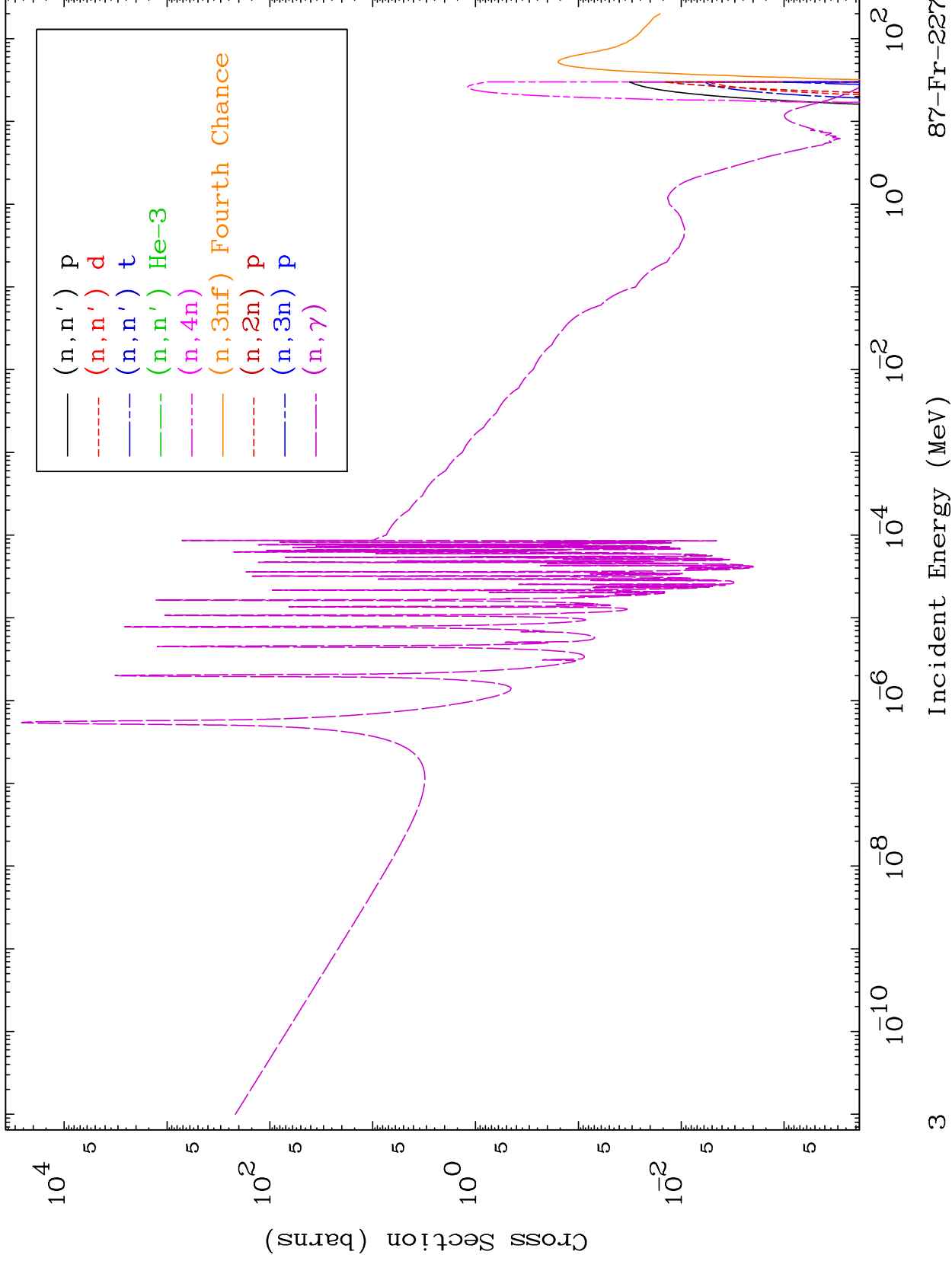


MAT 8770

Neutron Absorption
293 Kelvin Cross Sections

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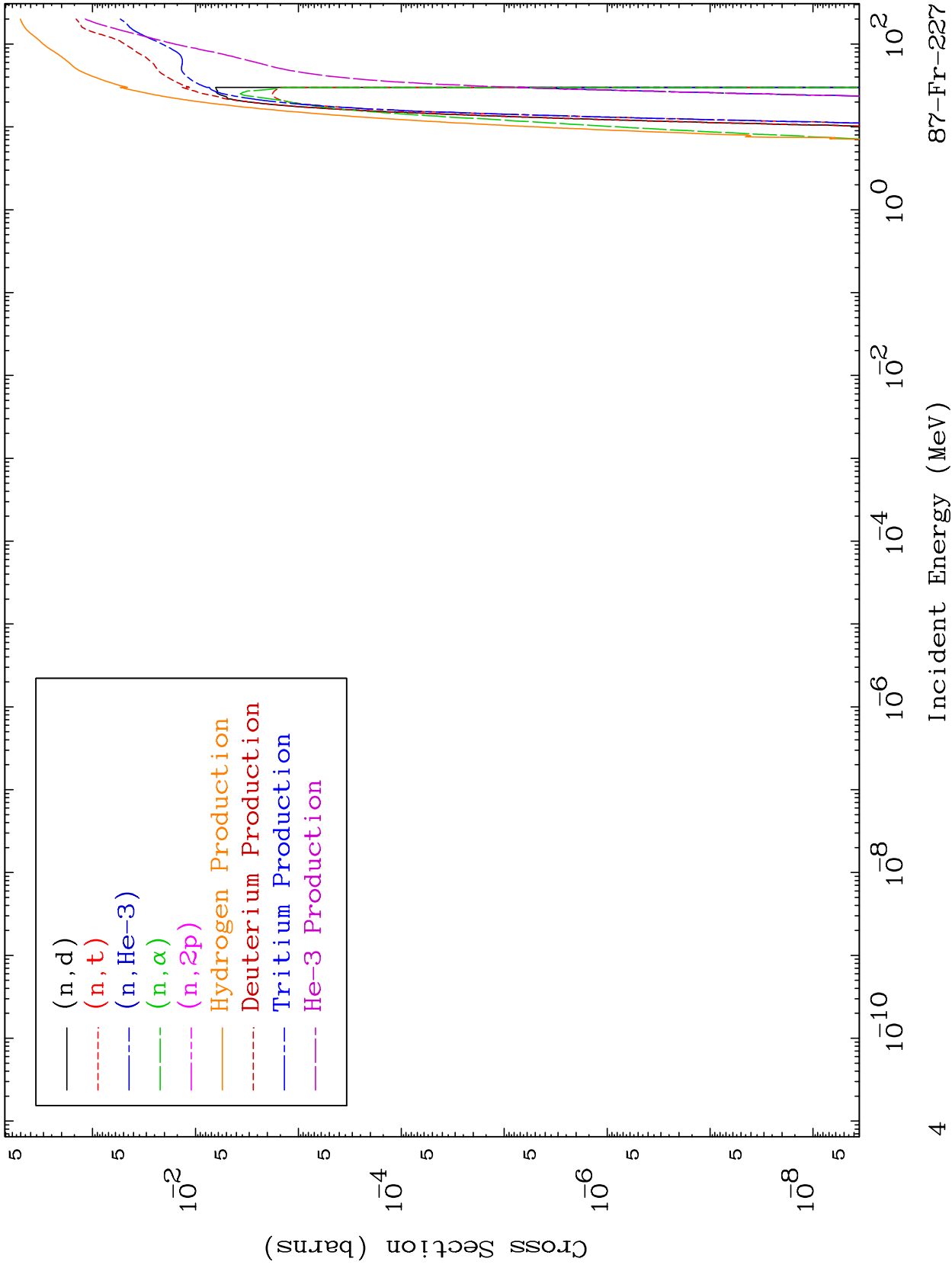




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Neutron Absorption
293 Kelvin Cross Sections

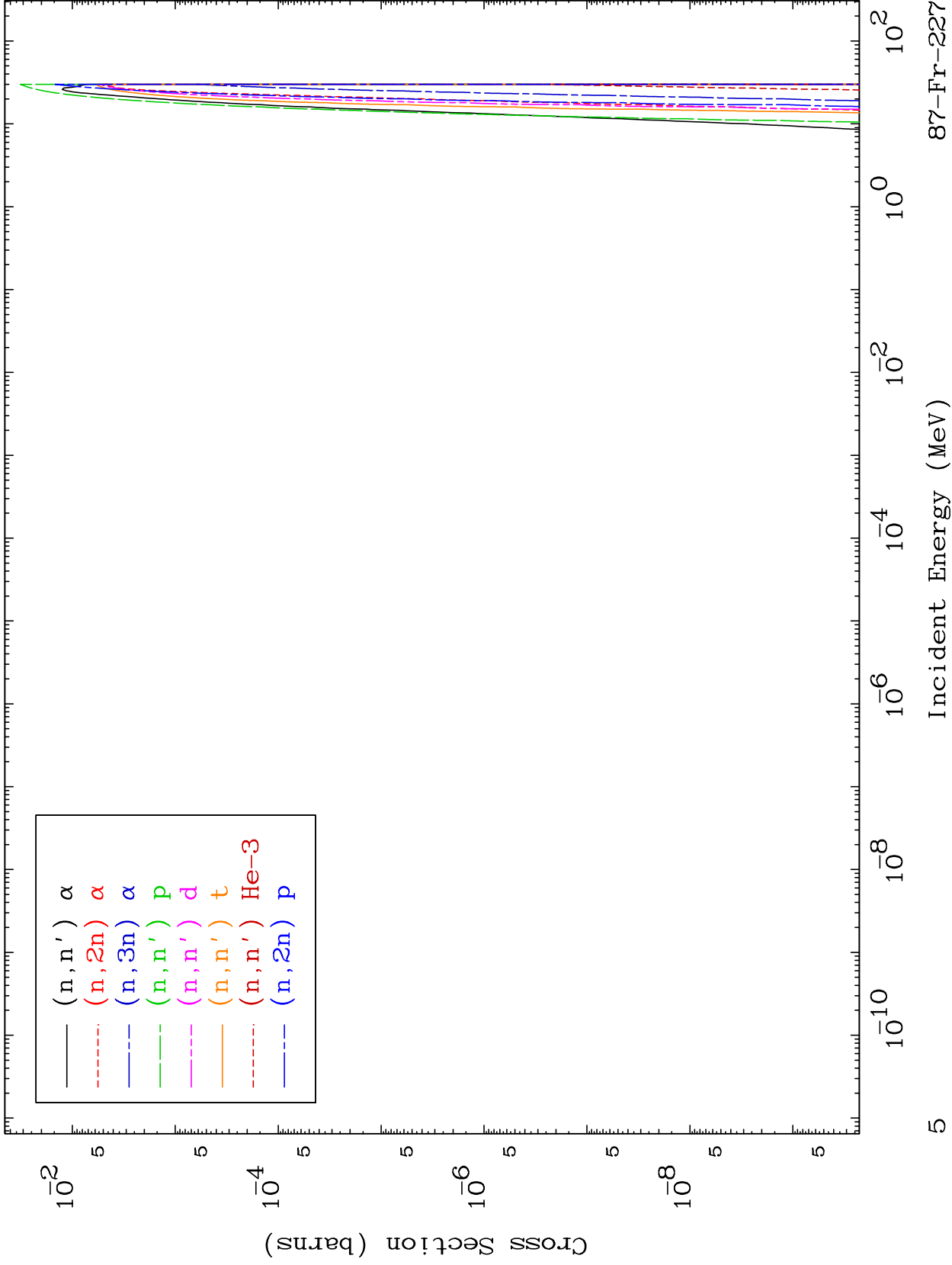
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Charged Particle
293 Kelvin Cross Sections

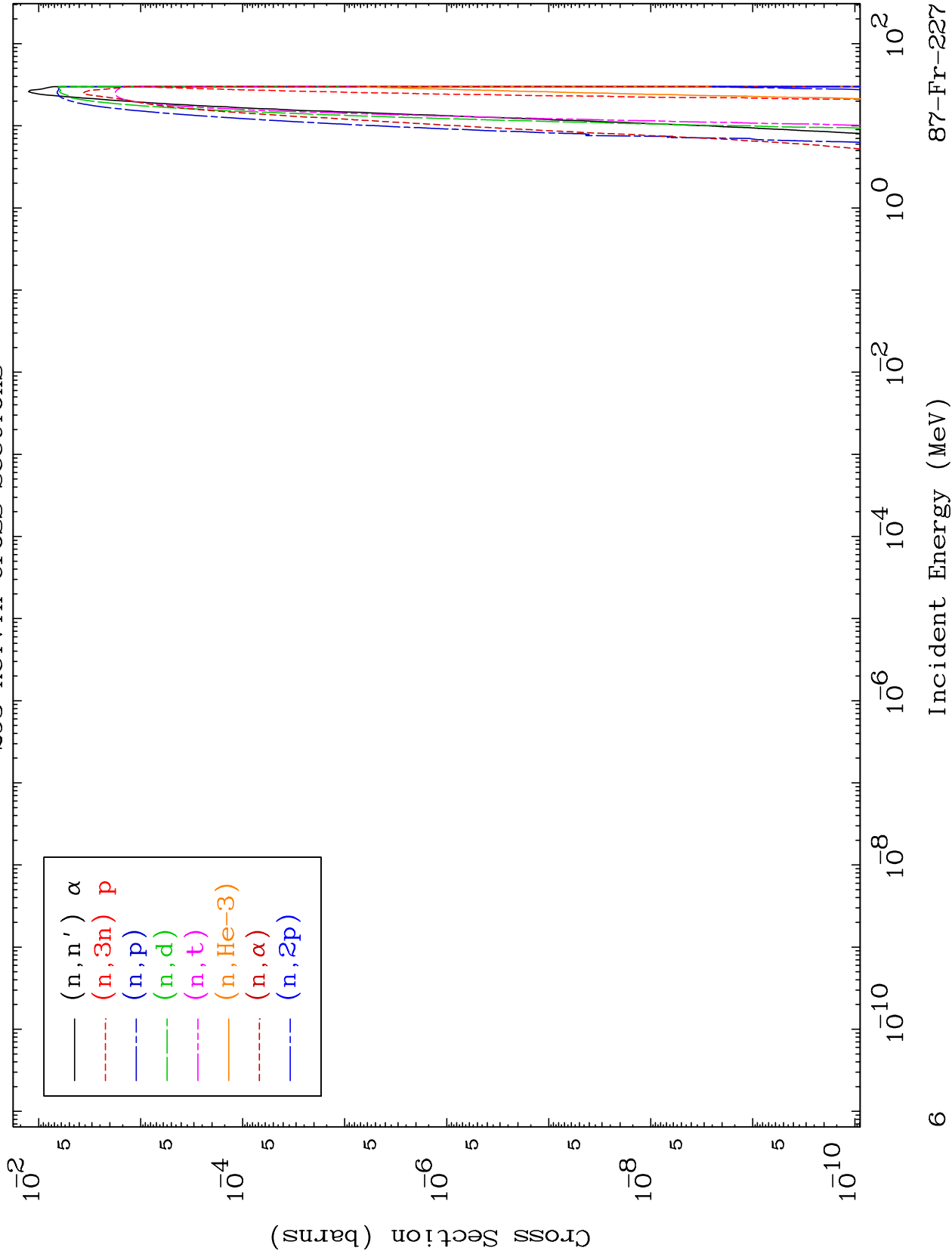
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Charged Particle
293 Kelvin Cross Sections

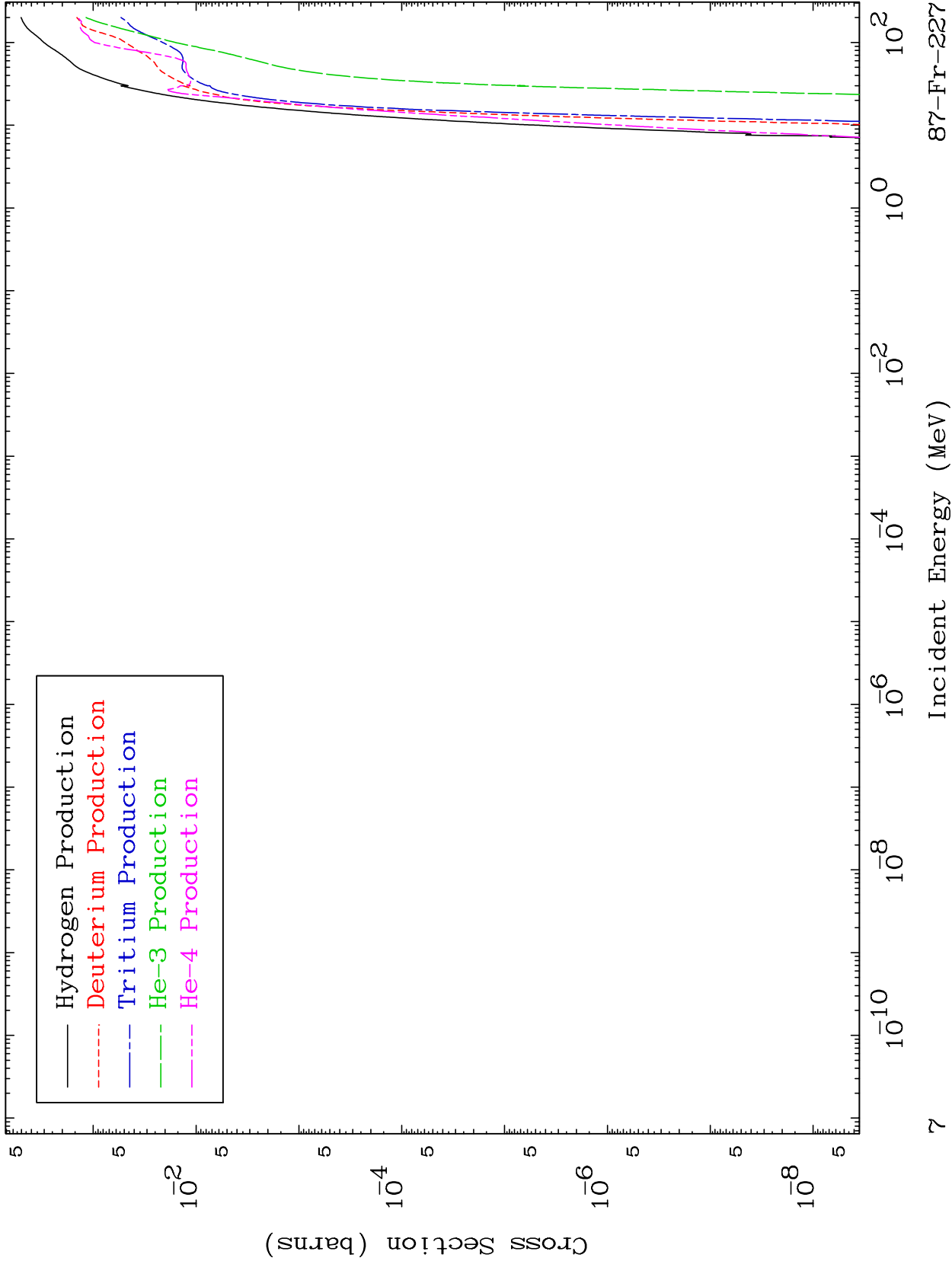
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Particle Production
293 Kelvin Cross Sections

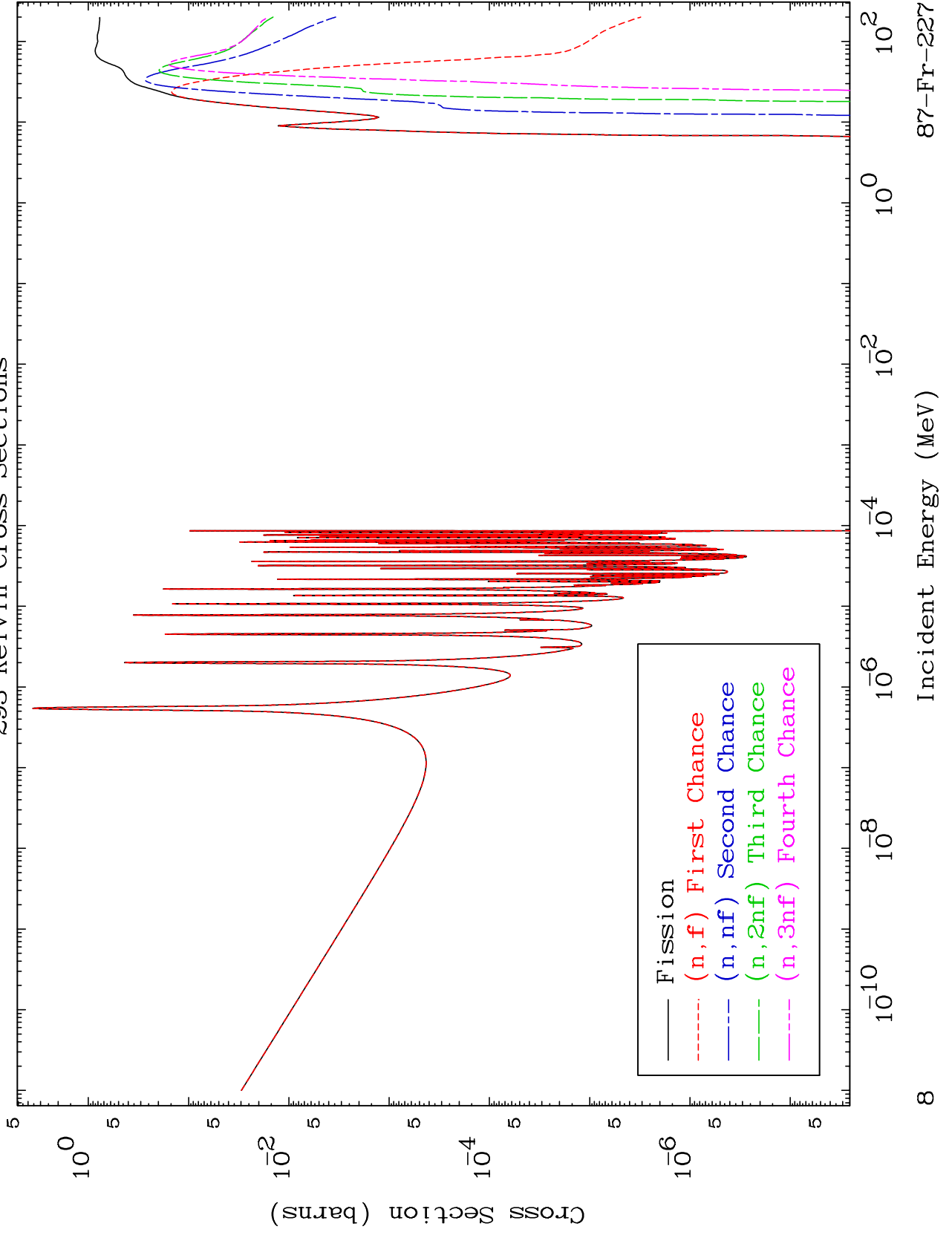
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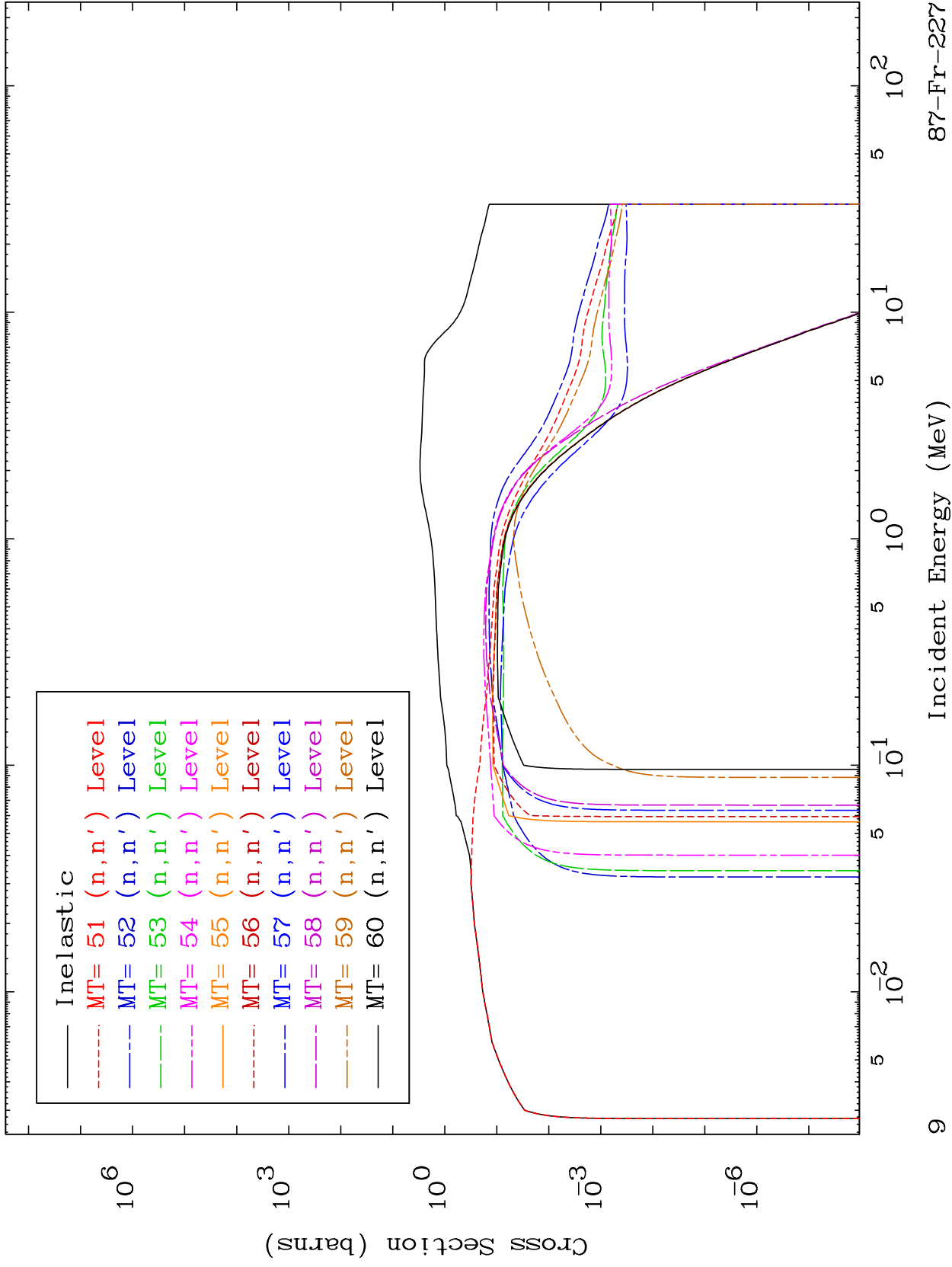


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Fission
293 Kelvin Cross Sections

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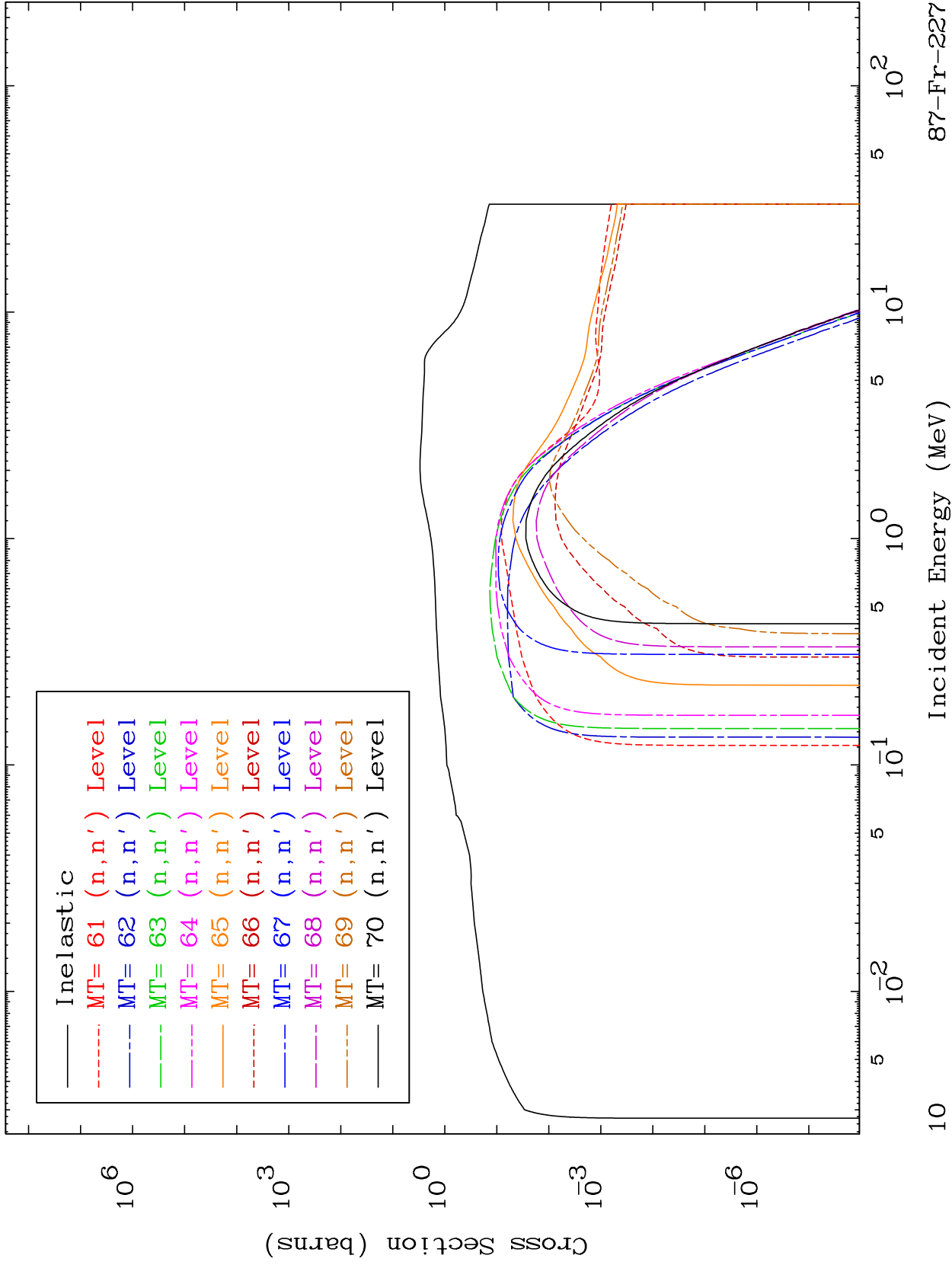




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(n,n') Levels
293 Kelvin Cross Sections

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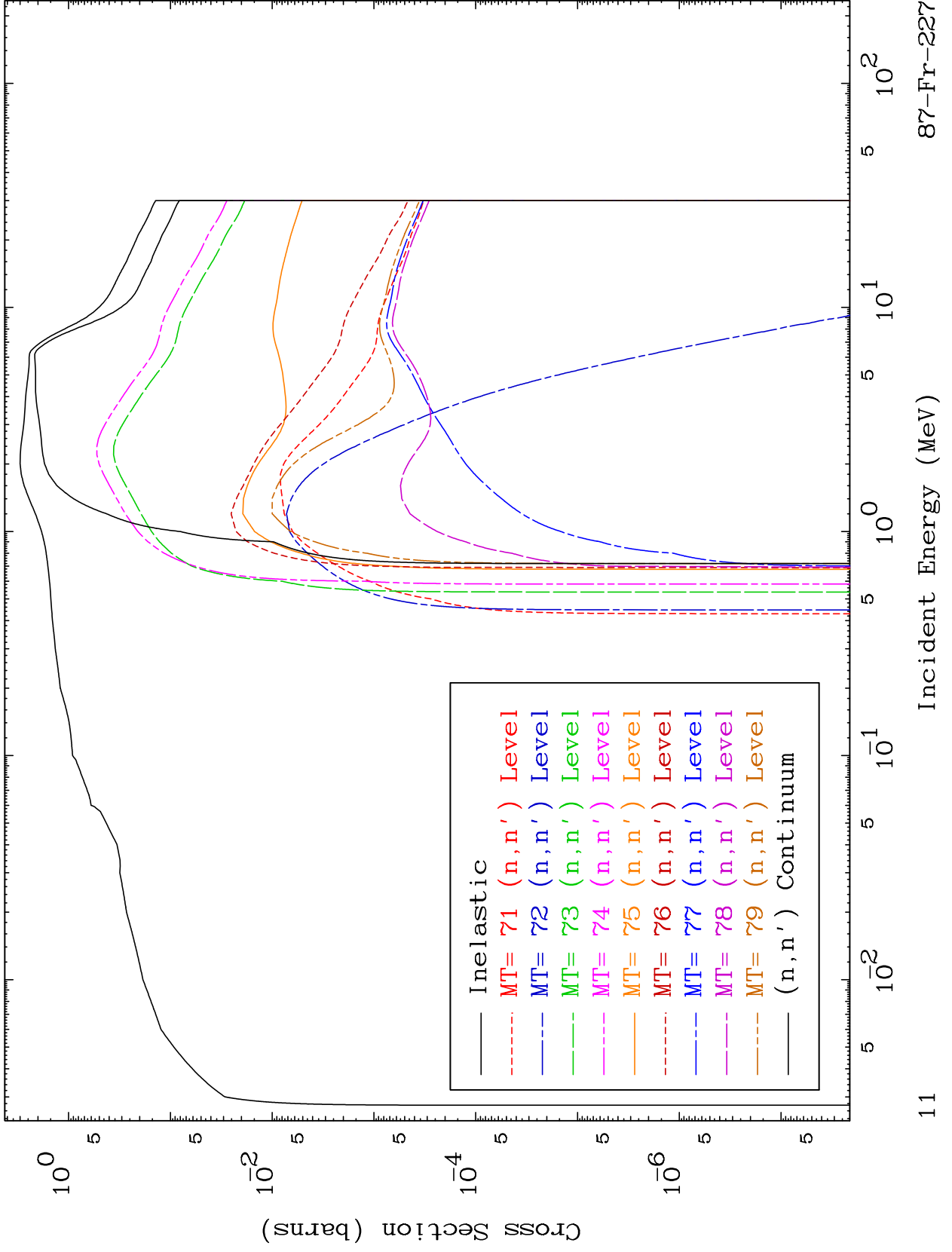
Incident Energy (MeV)

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MAT 8770

293 Kelvin Cross Sections
(n,n') Levels

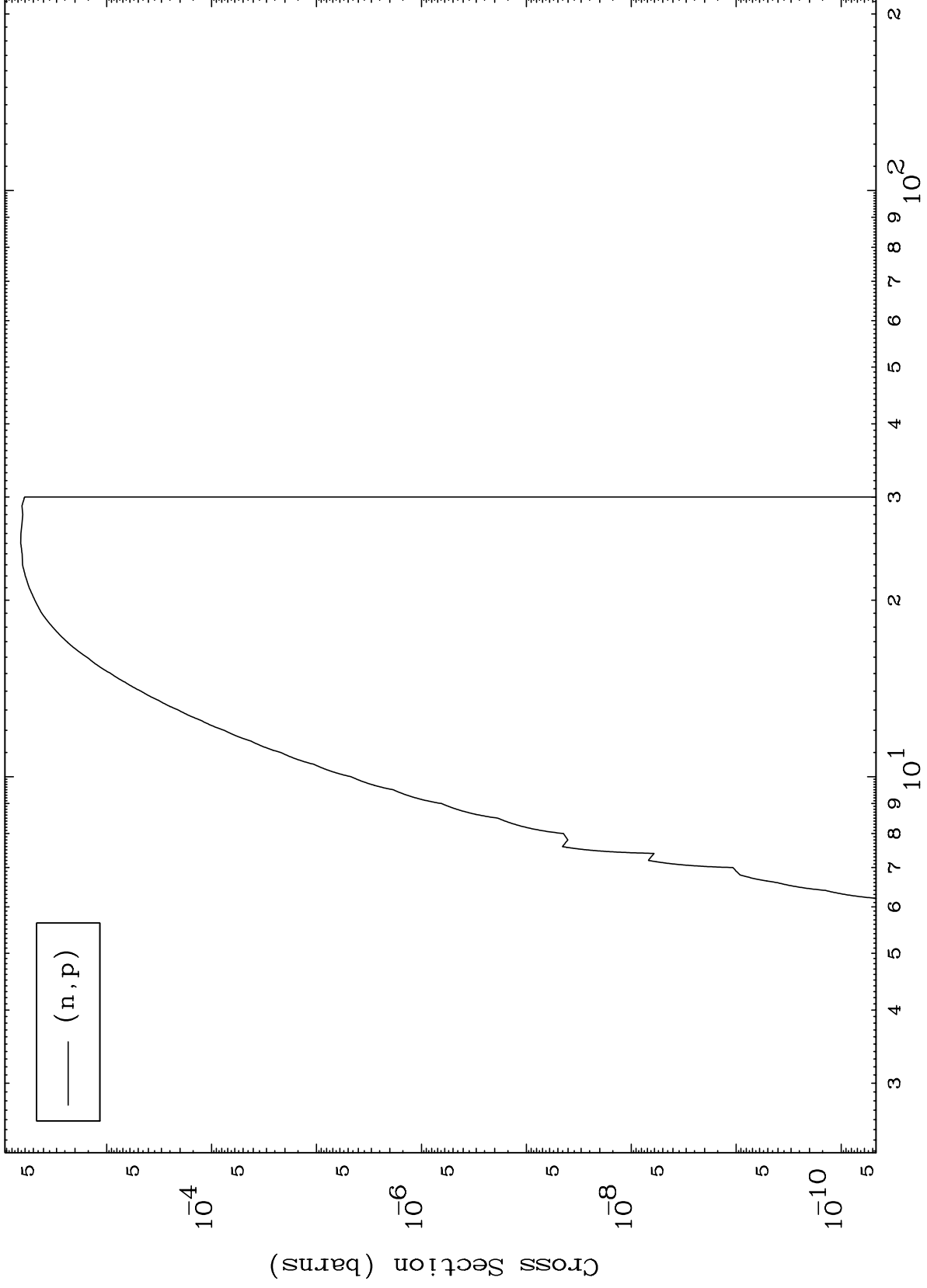
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(n,p) Levels
293 Kelvin Cross Sections

87-Fr-227



12

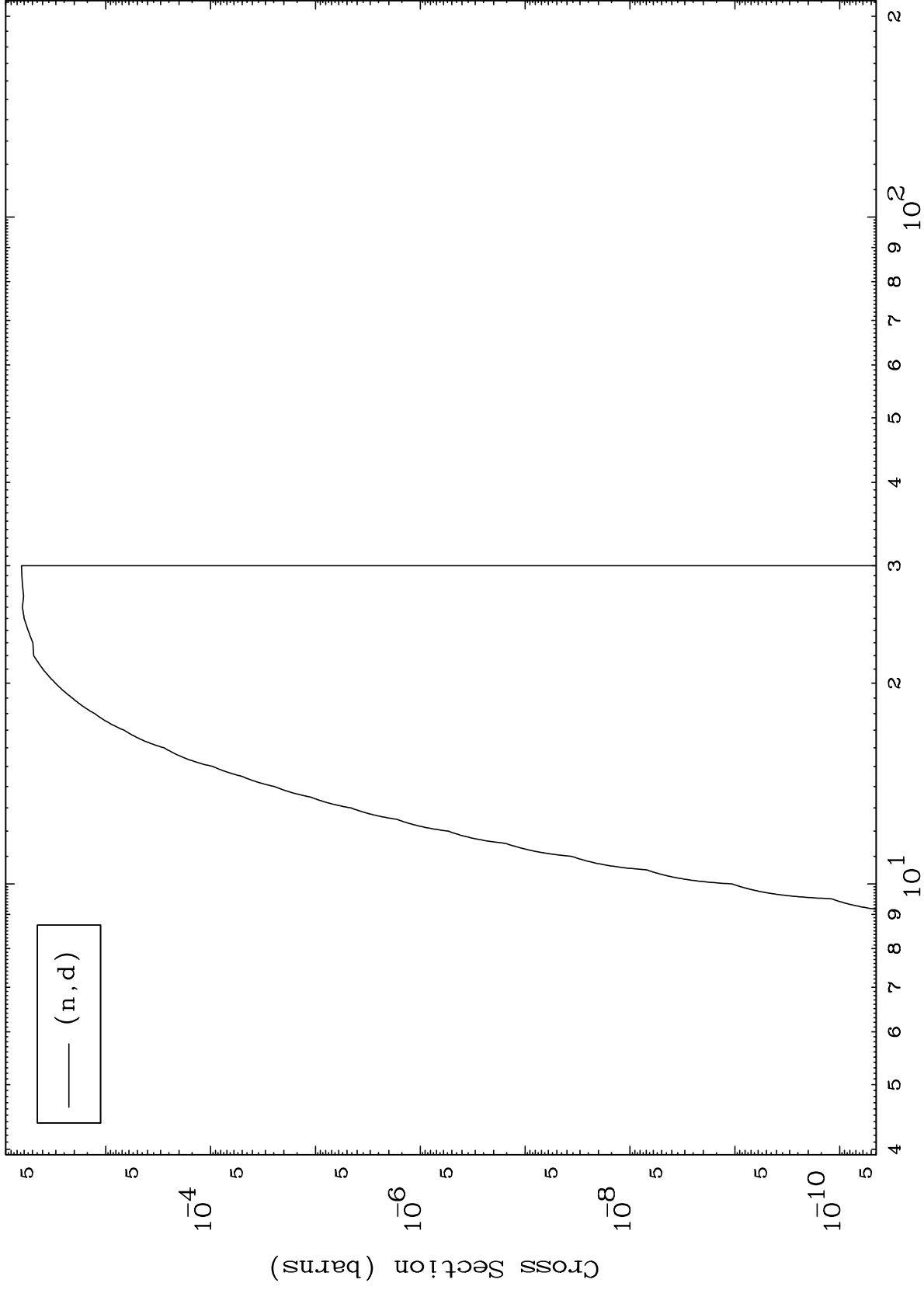
Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

87-Fr-227



13

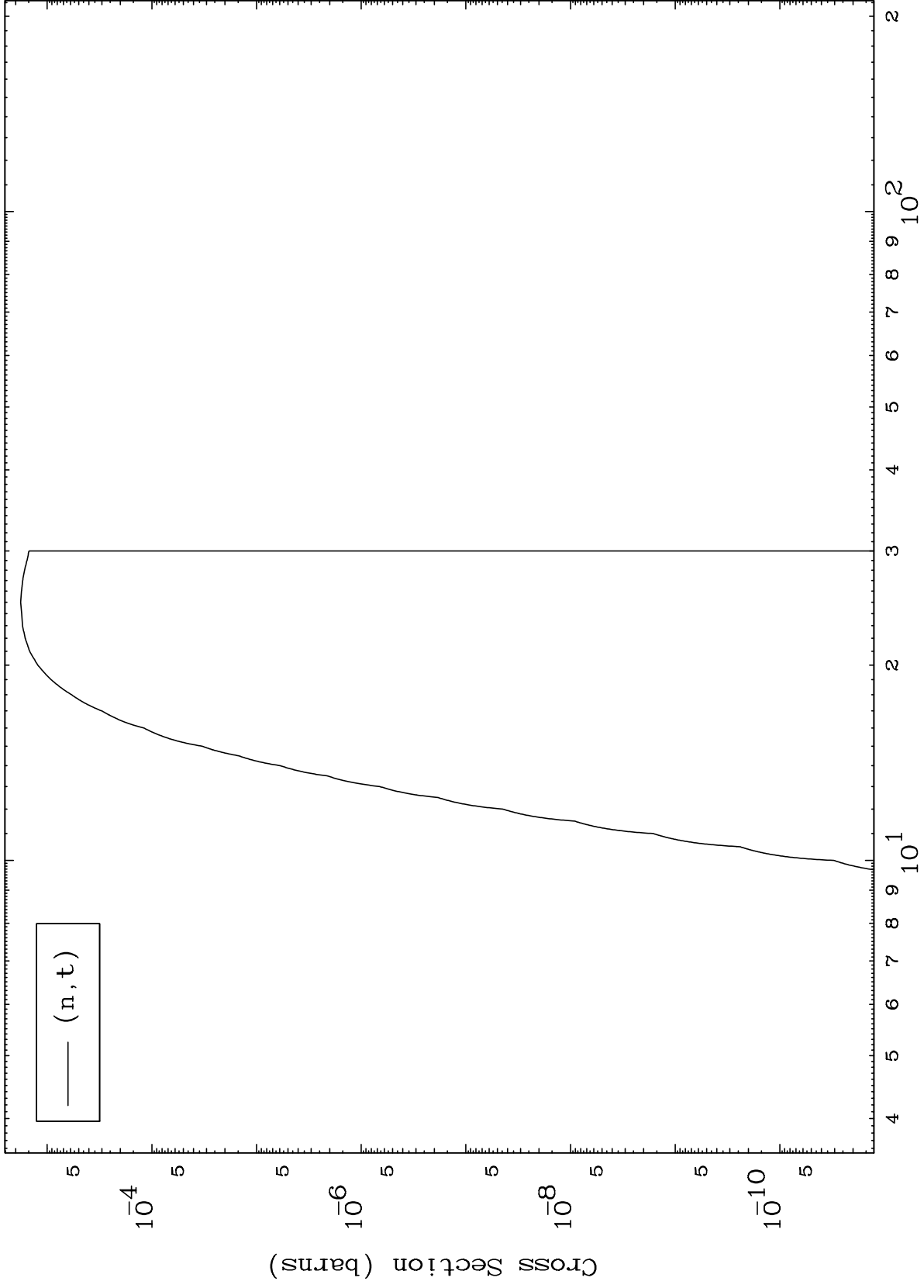
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

87-Fr-227



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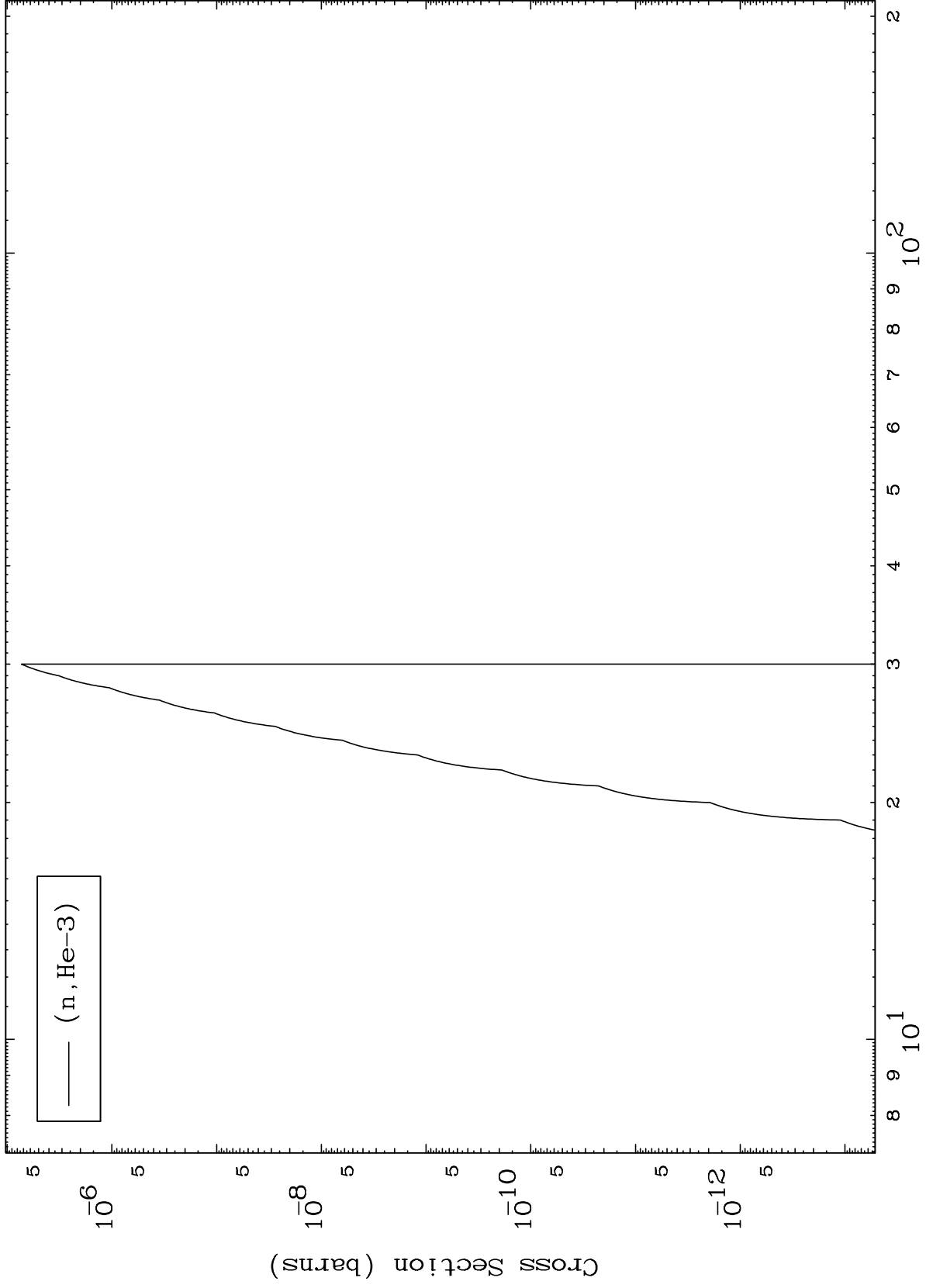
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

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15

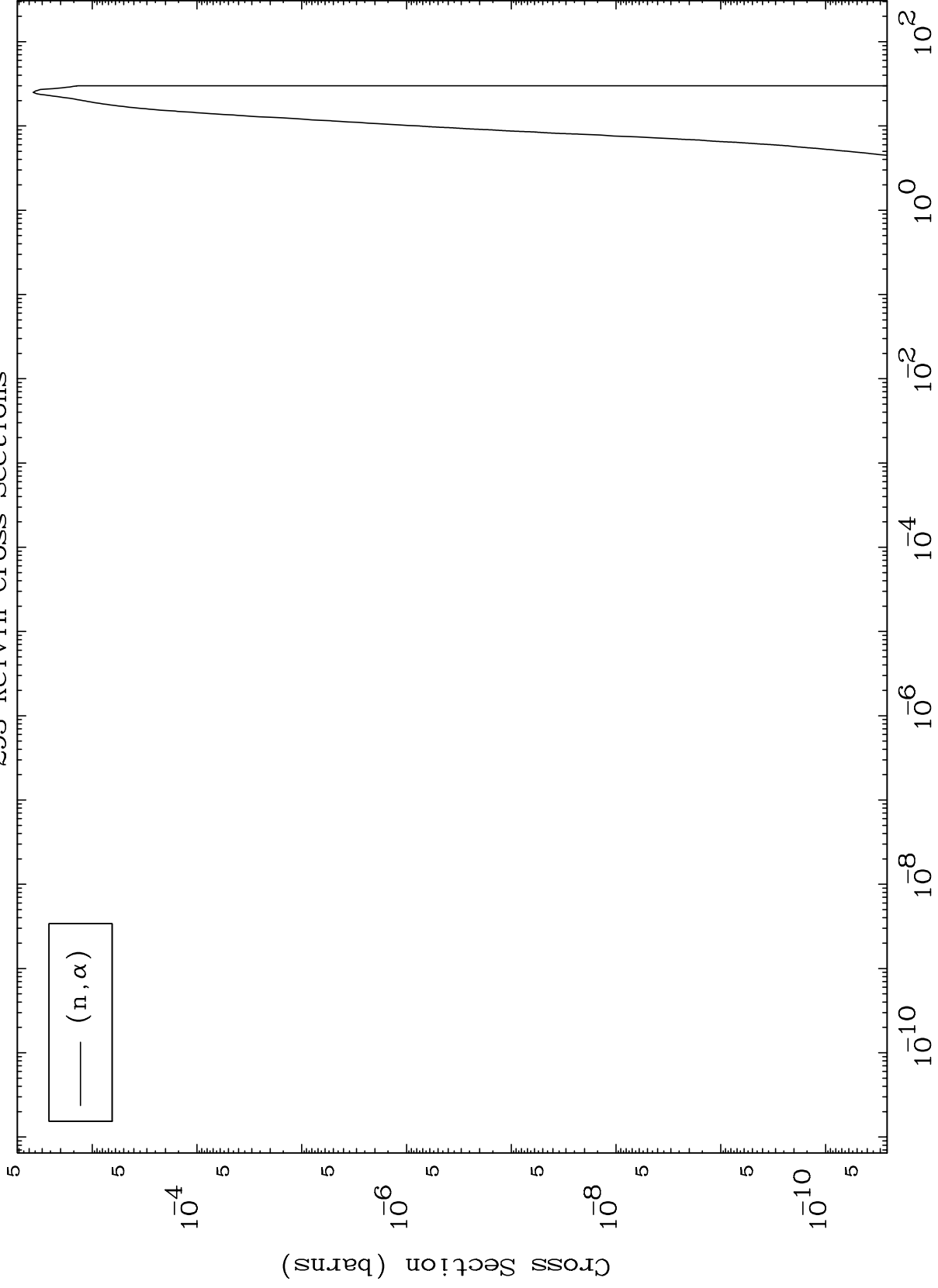
Incident Energy (MeV)

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MAT 8770

(n, α) Levels
293 Kelvin Cross Sections

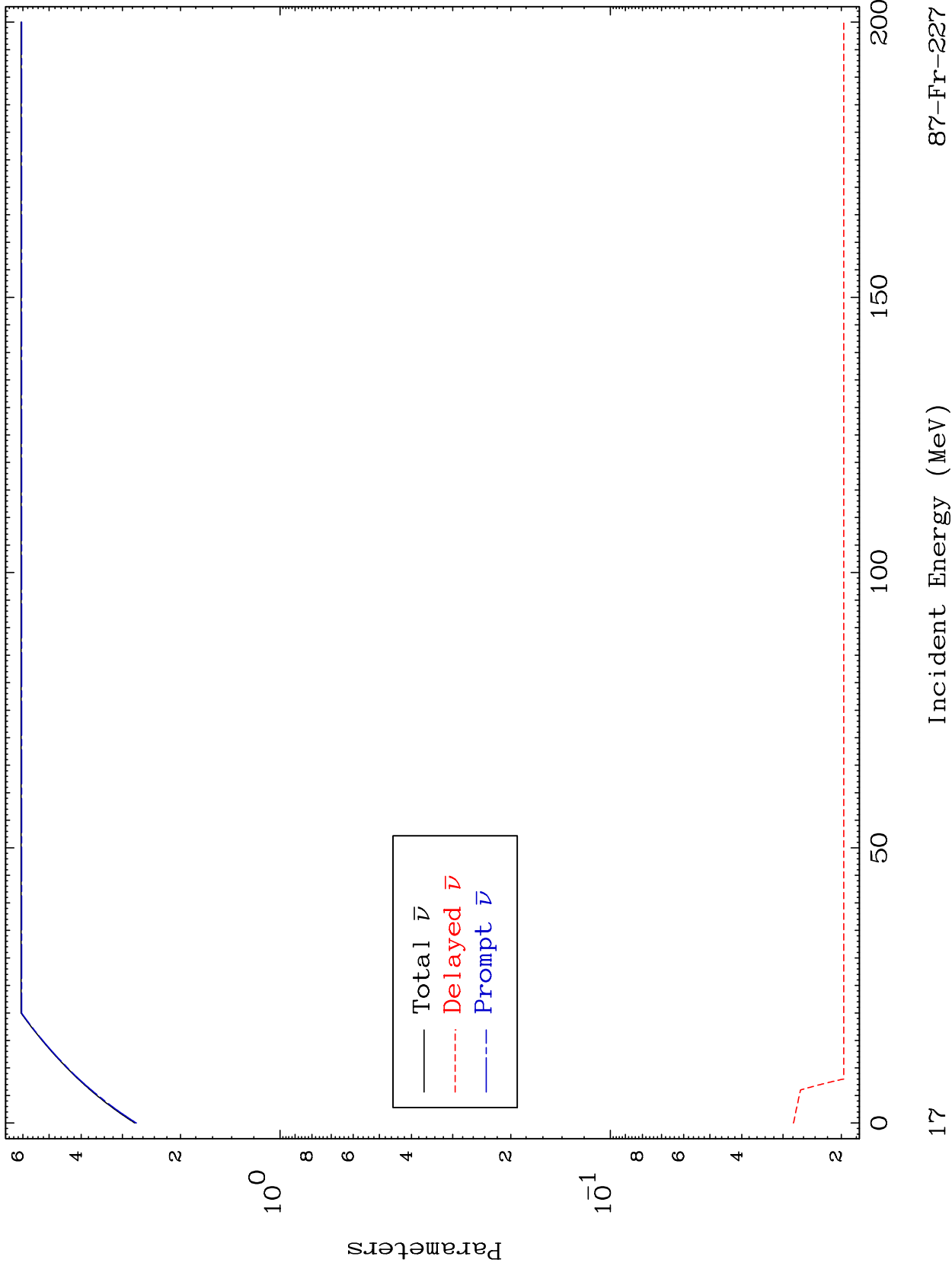
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Energy Release
Parameters

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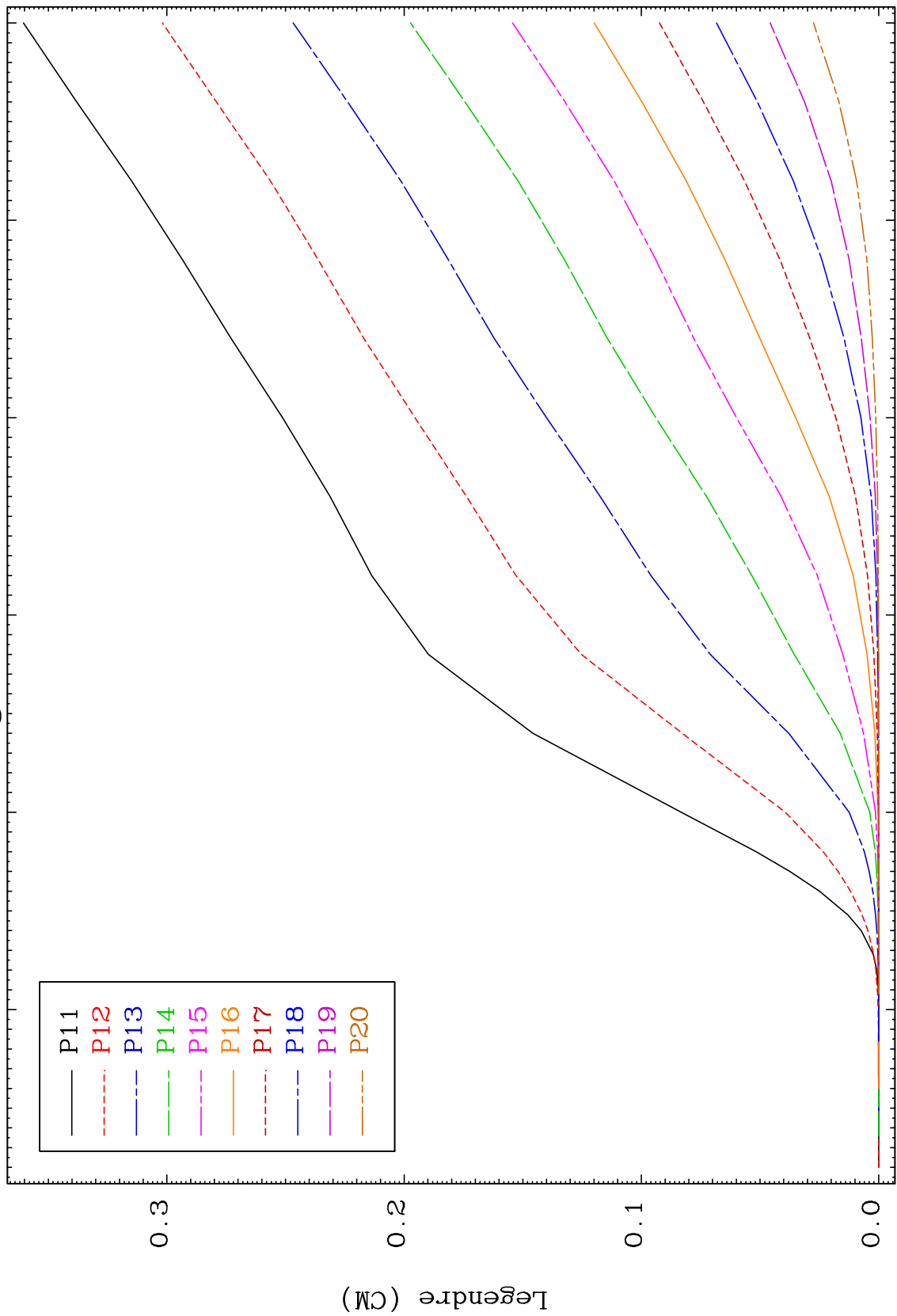
Incident Energy (MeV)

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Elastic Legendre Coefficients

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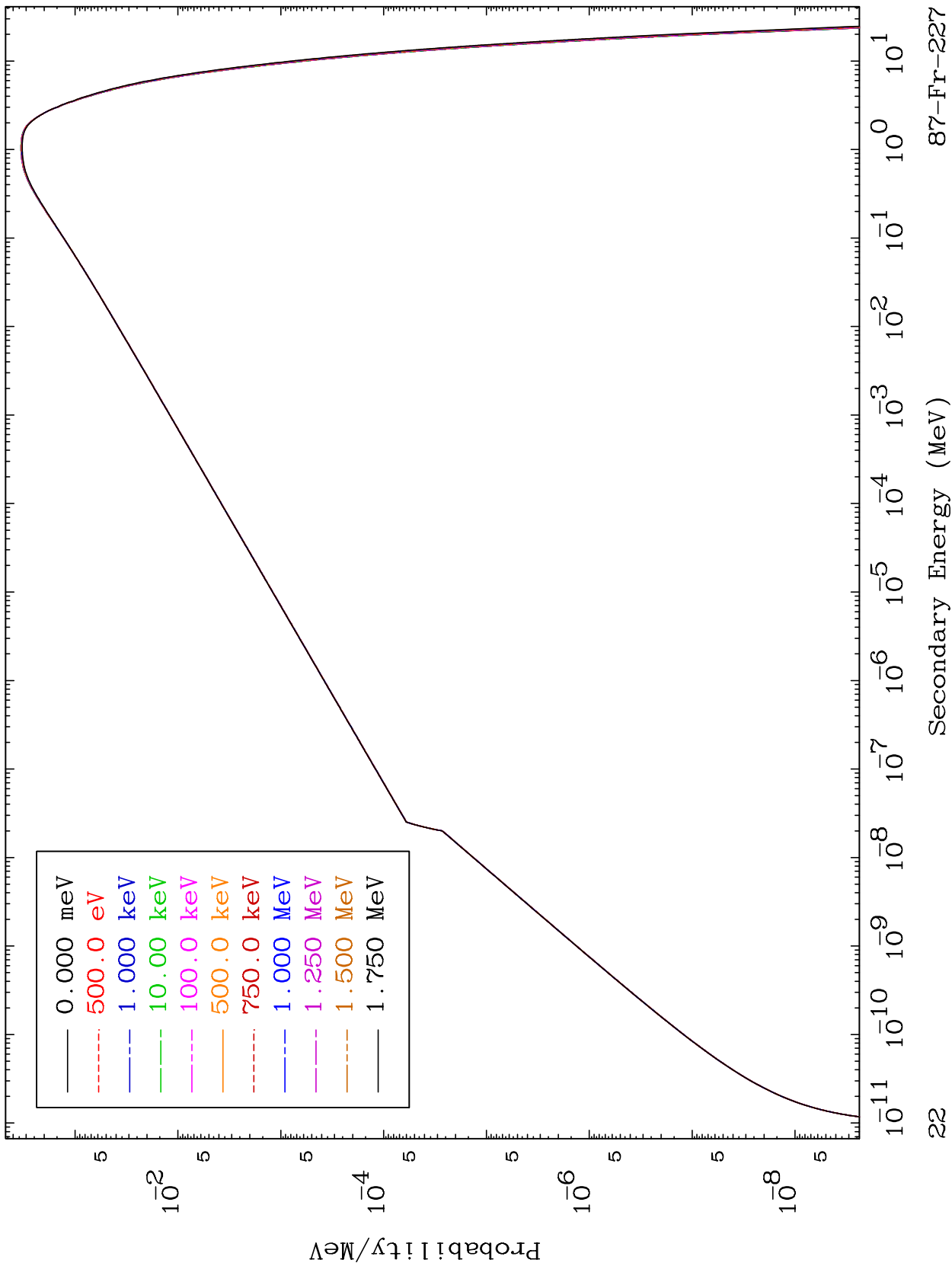
Incident Energy (MeV)

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Fission Energy Distributions

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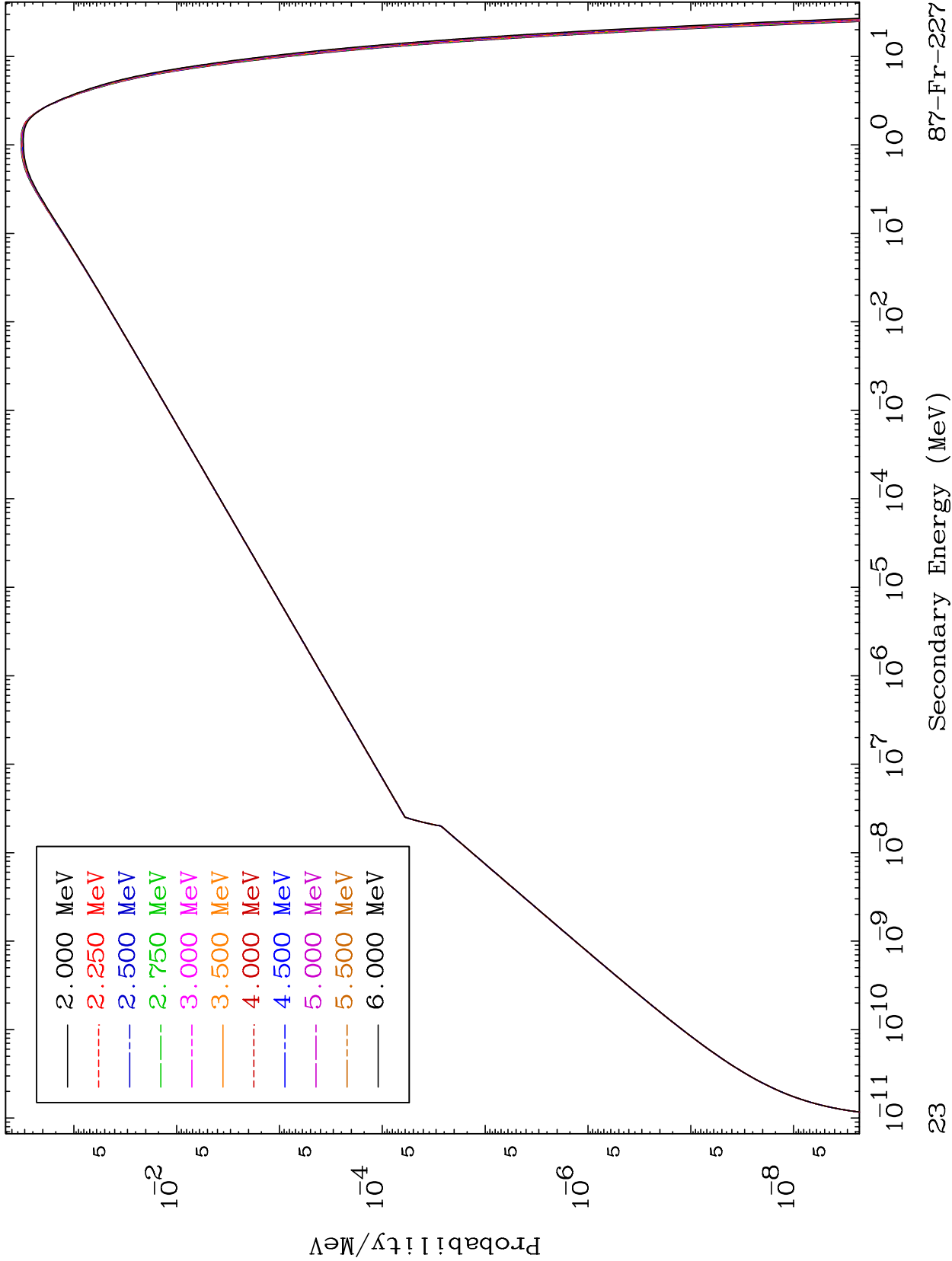
Secondary Energy (MeV)

22

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Fission Energy Distributions

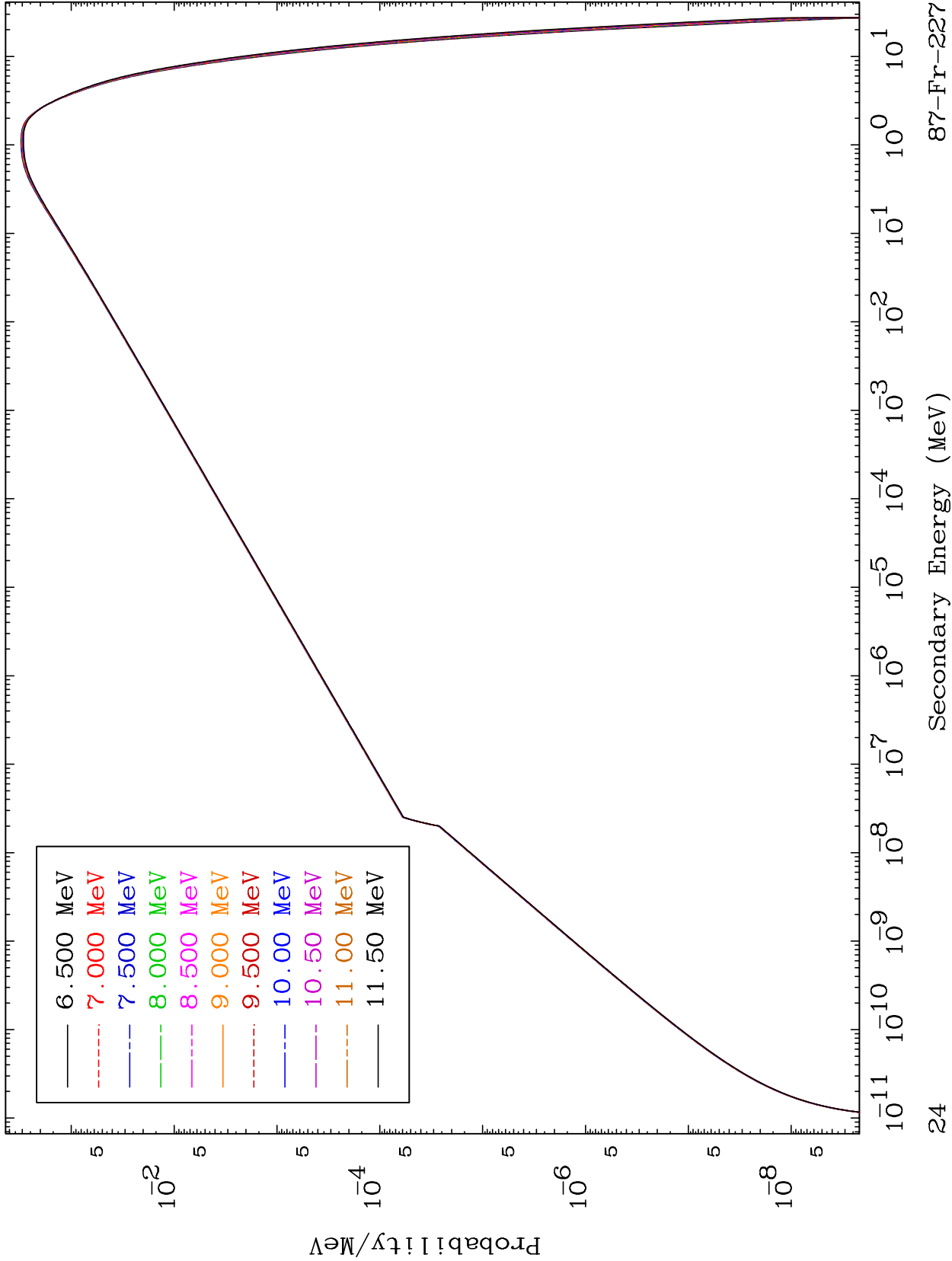
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Fission Energy Distributions

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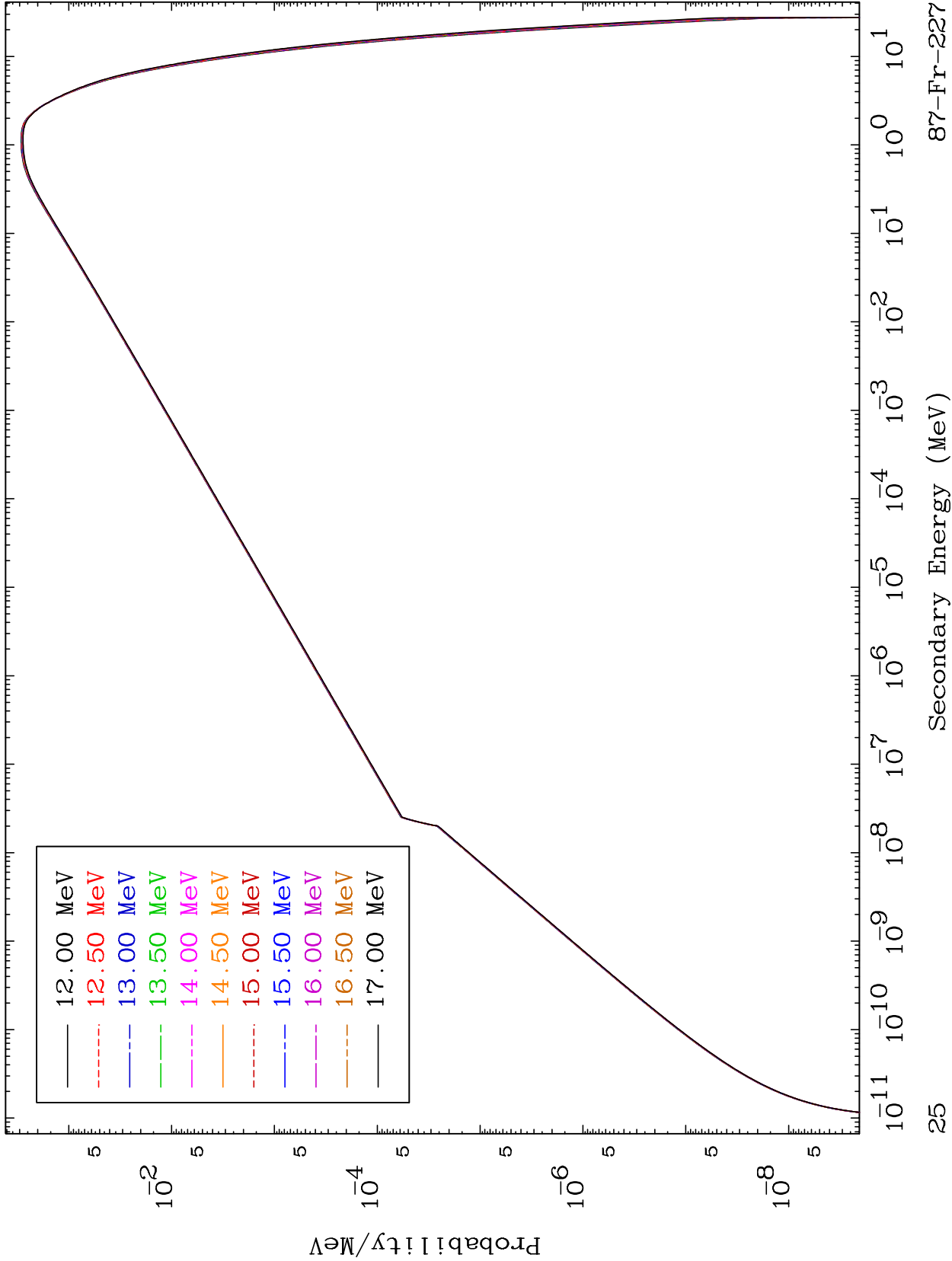
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Secondary Energy (MeV)

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Fission Energy Distributions

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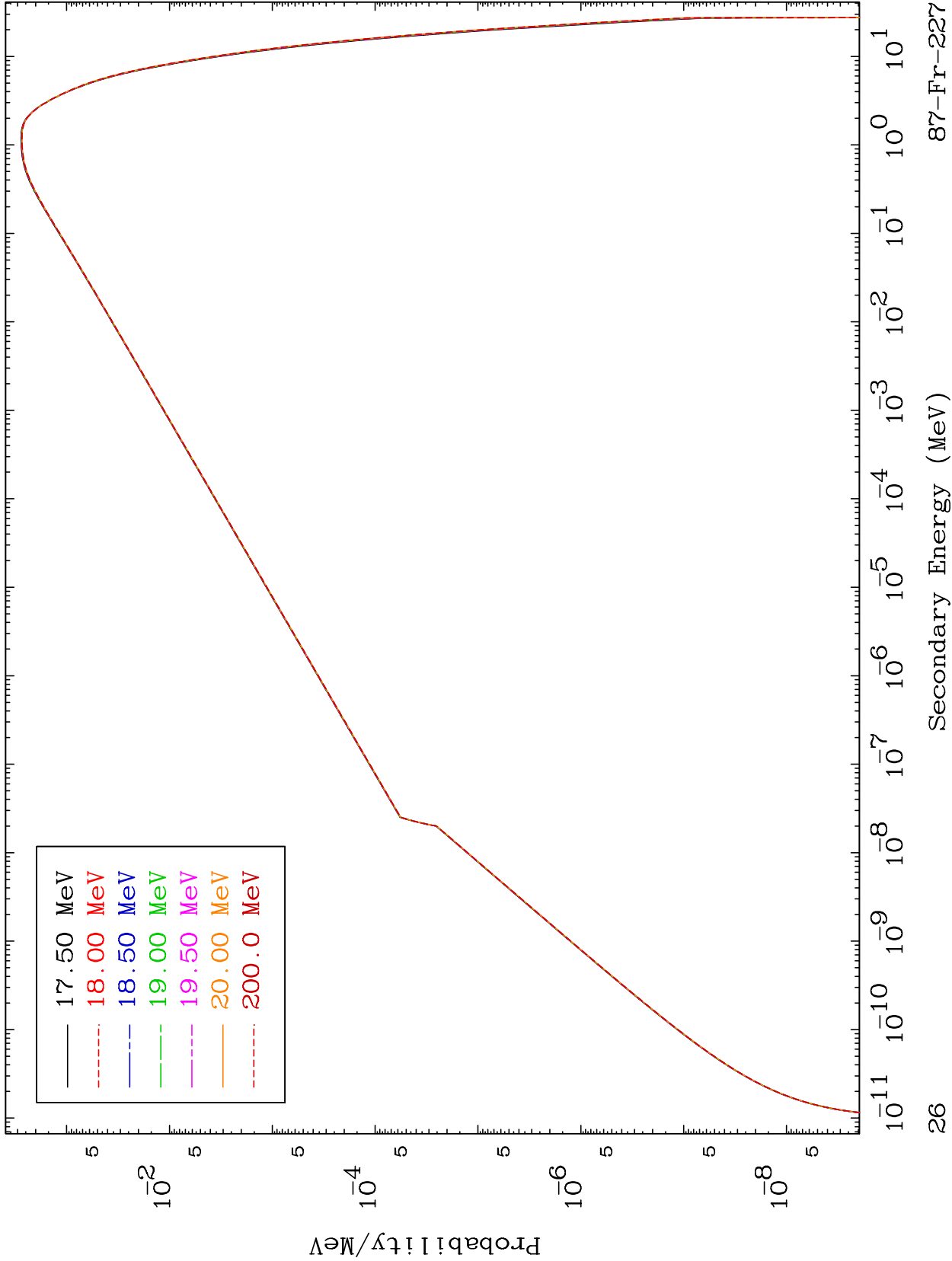


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Secondary Energy (MeV)

25

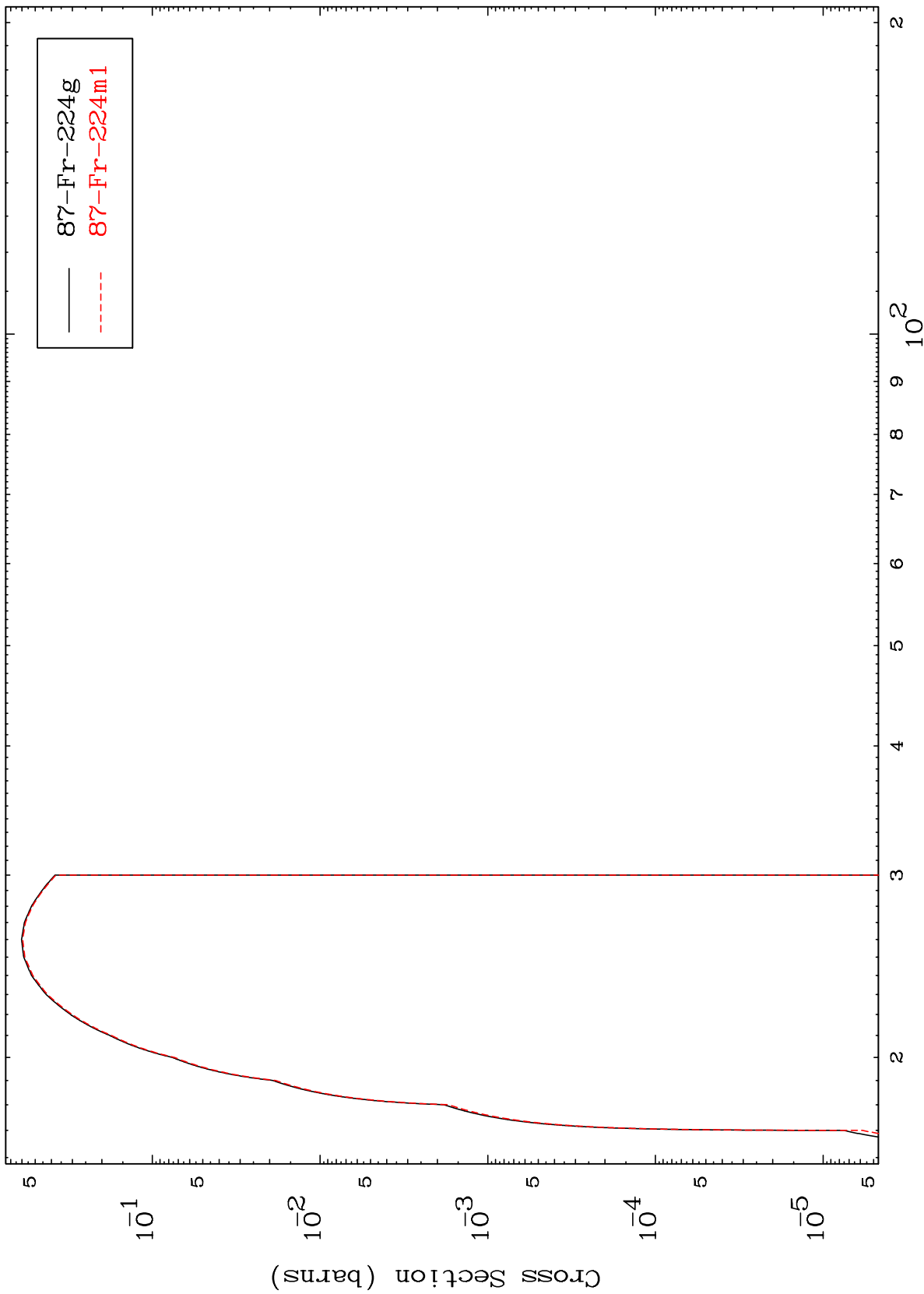
Fission Energy Distributions



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(n,4n)
Radionuclide Production Cross Section



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Incident Energy (MeV)

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