

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

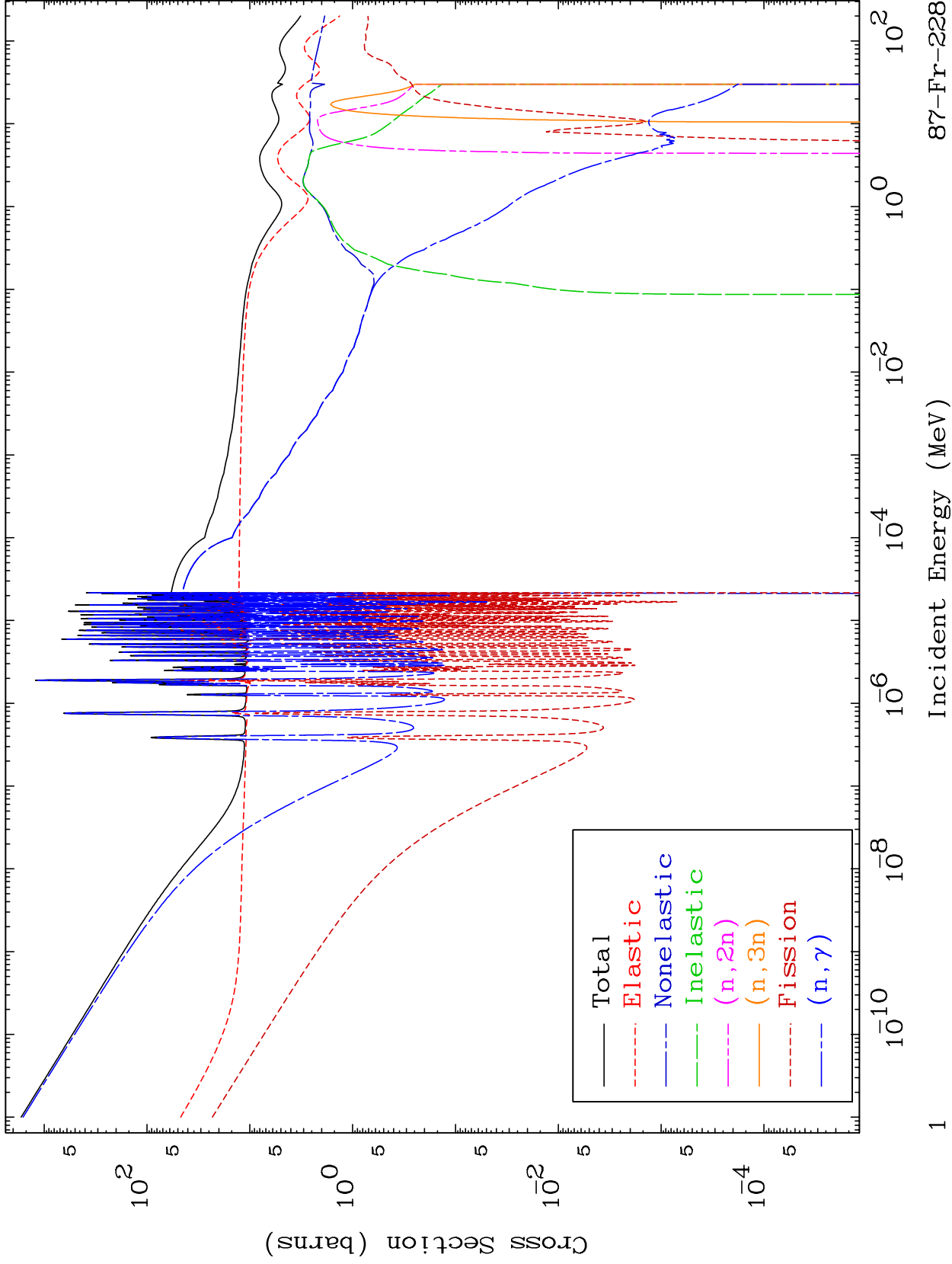
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Press Mouse Button to Start

MAT 8773

Neutron Major
293 Kelvin Cross Sections

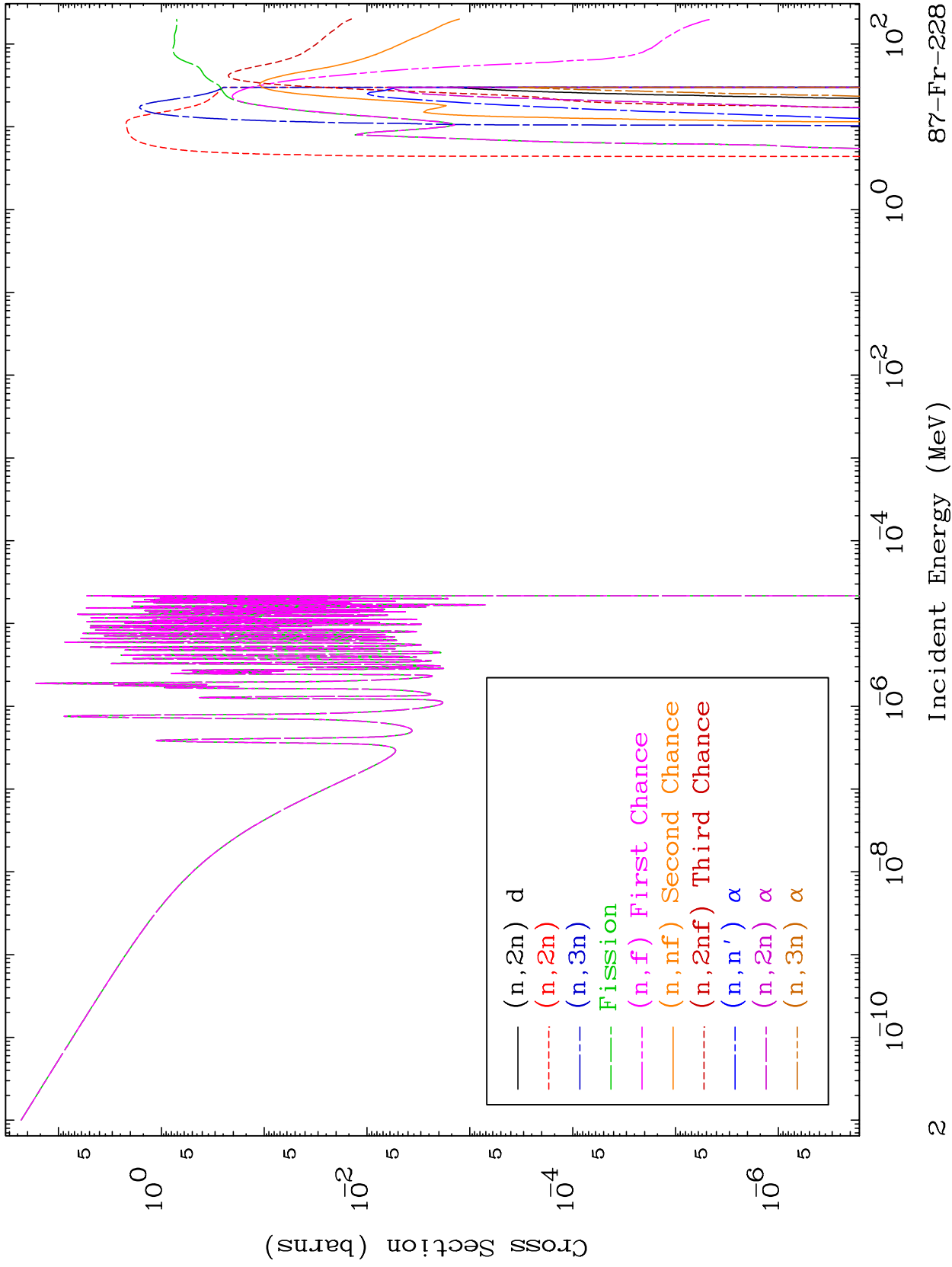
87-Fr-228



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Neutron Absorption
293 Kelvin Cross Sections

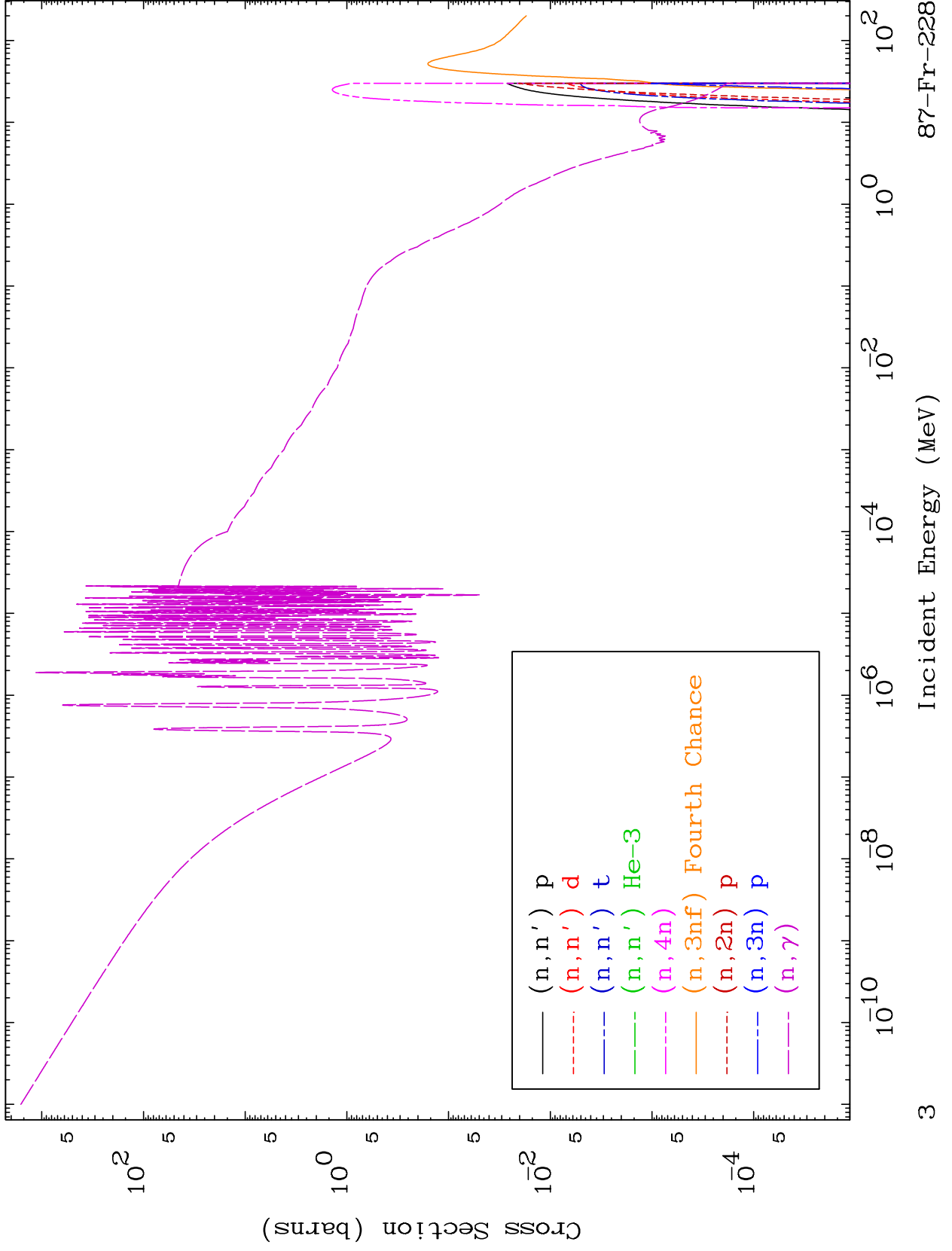
87-Fr-228



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Neutron Absorption
293 Kelvin Cross Sections

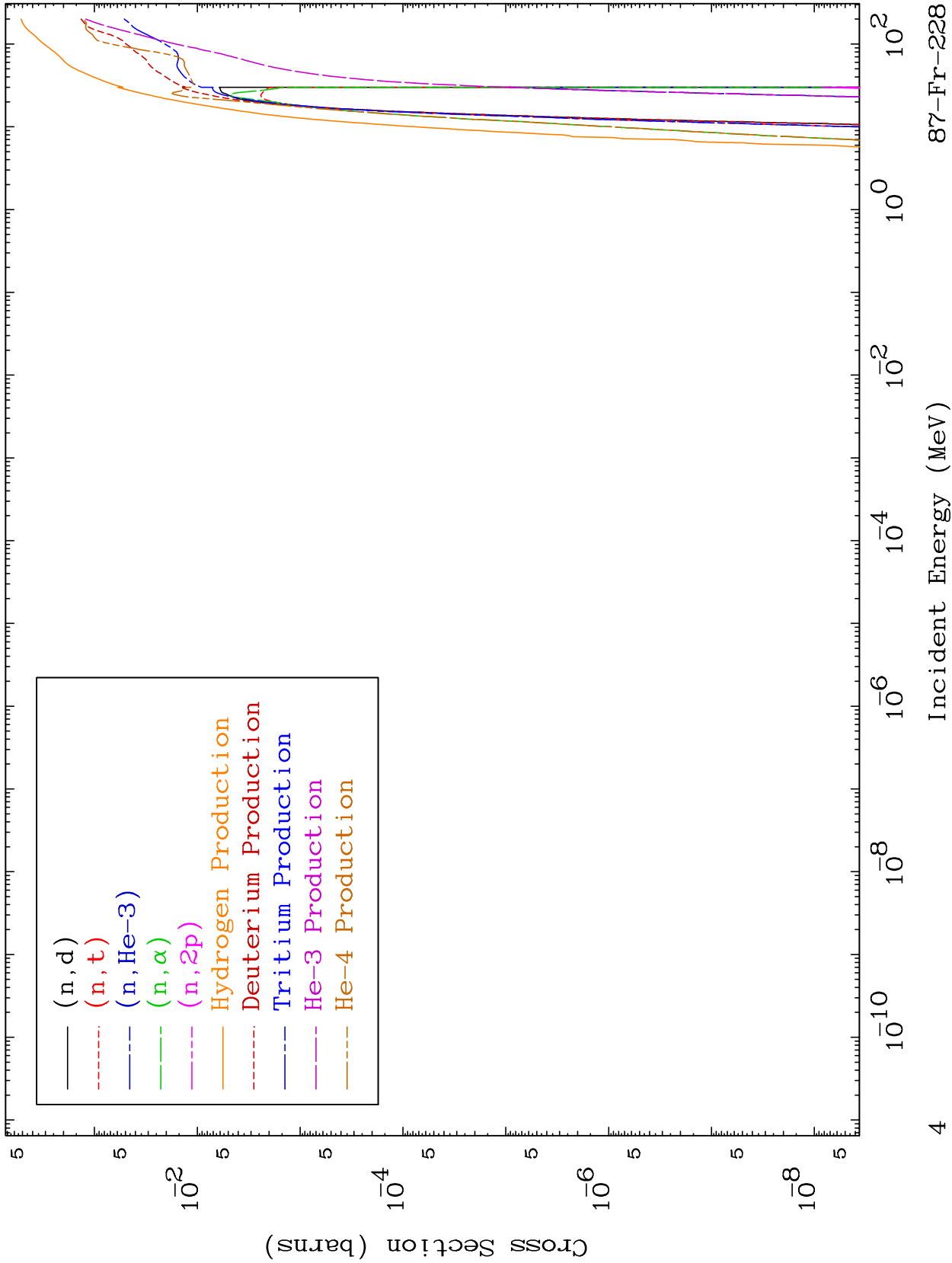
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Neutron Absorption
293 Kelvin Cross Sections

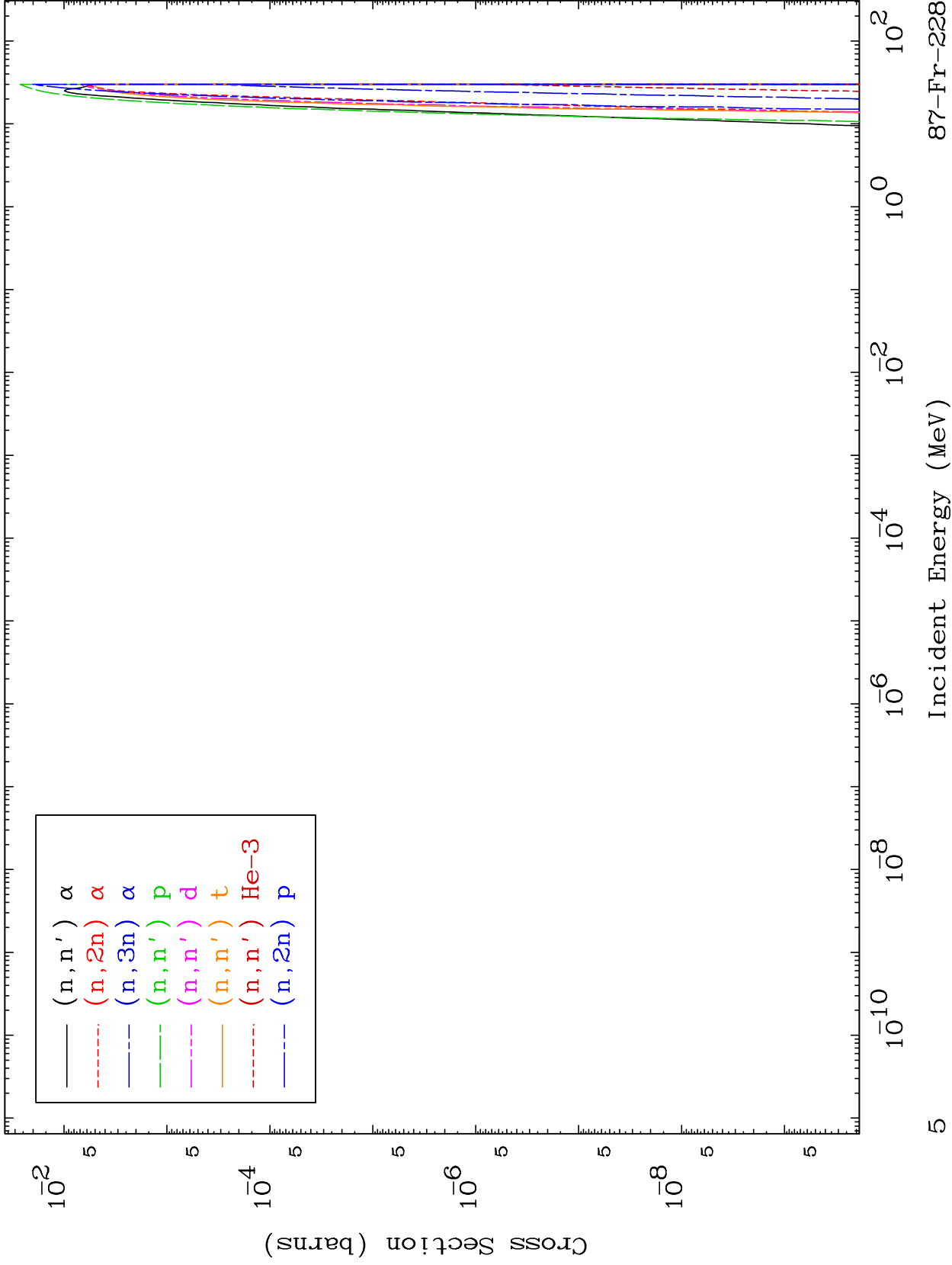
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Charged Particle
293 Kelvin Cross Sections

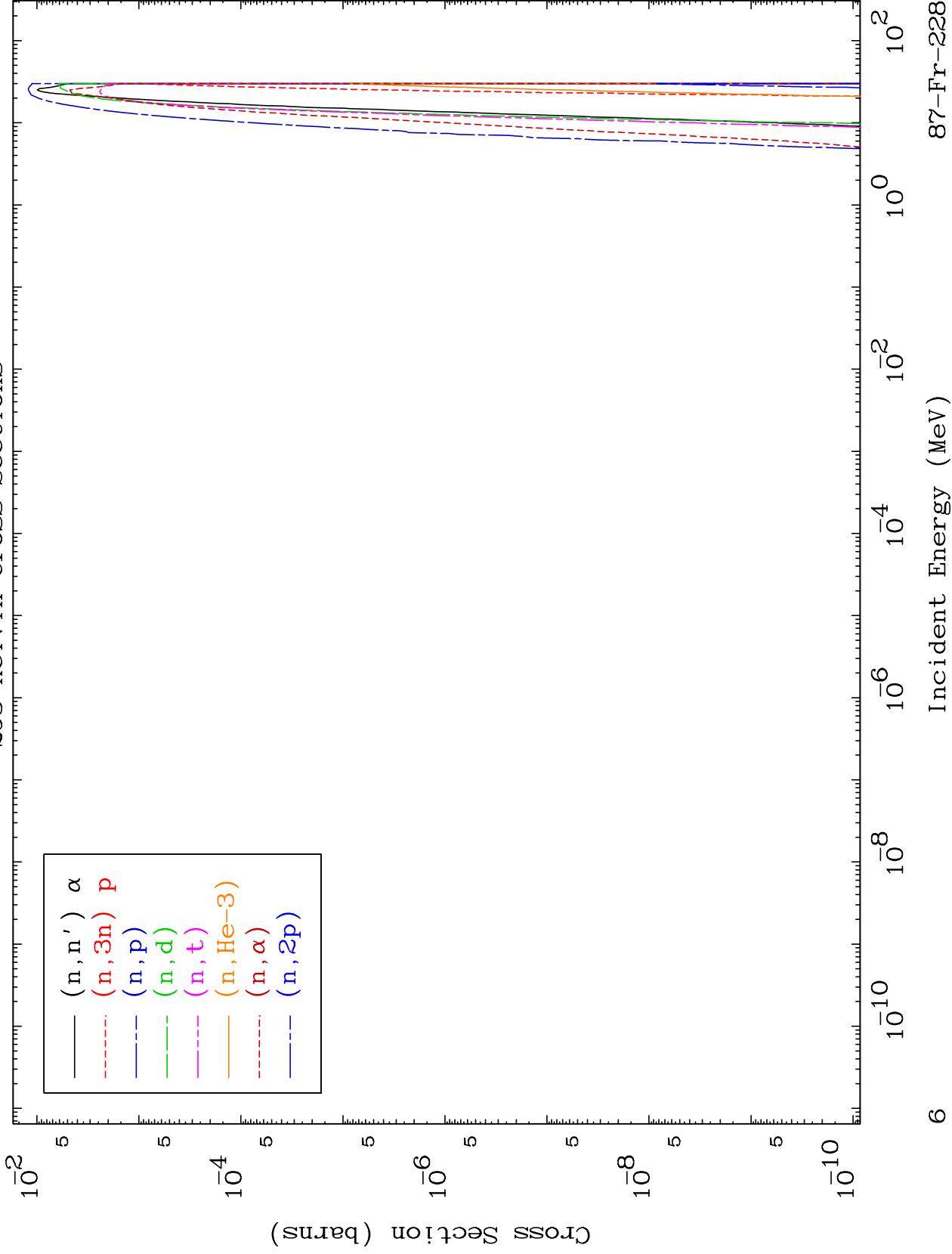
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Charged Particle
293 Kelvin Cross Sections

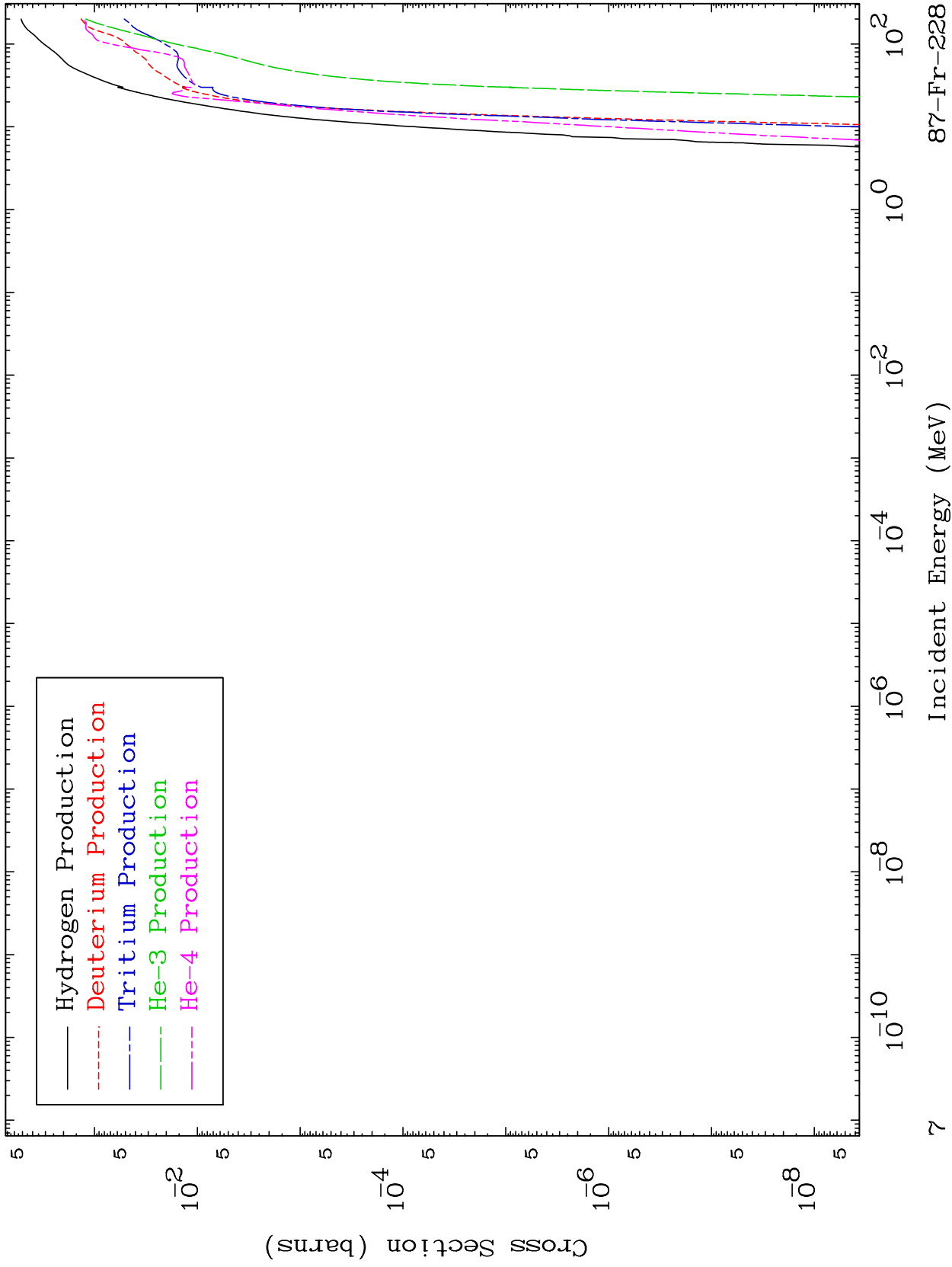
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Particle Production
293 Kelvin Cross Sections

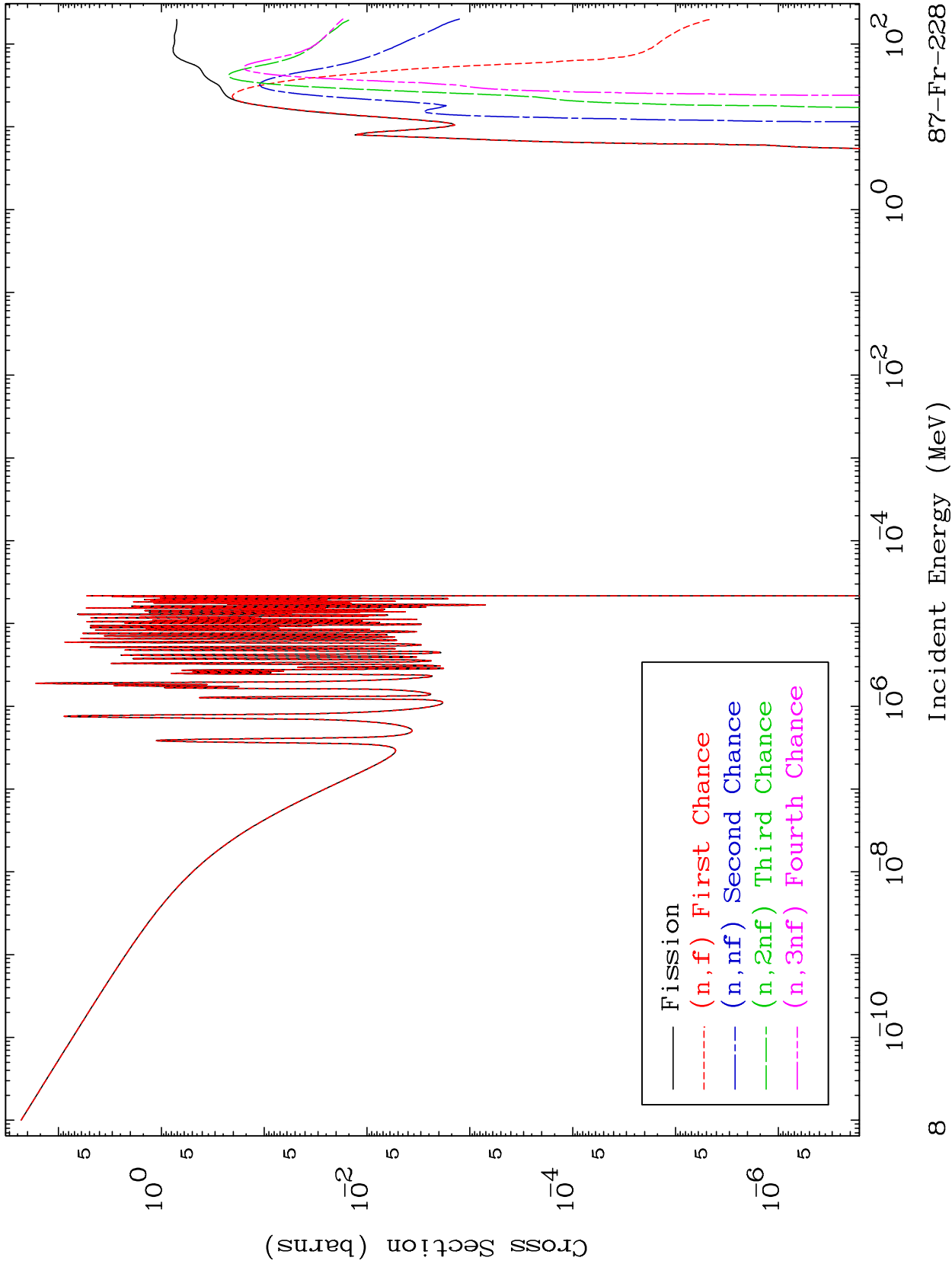
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Fission
293 Kelvin Cross Sections

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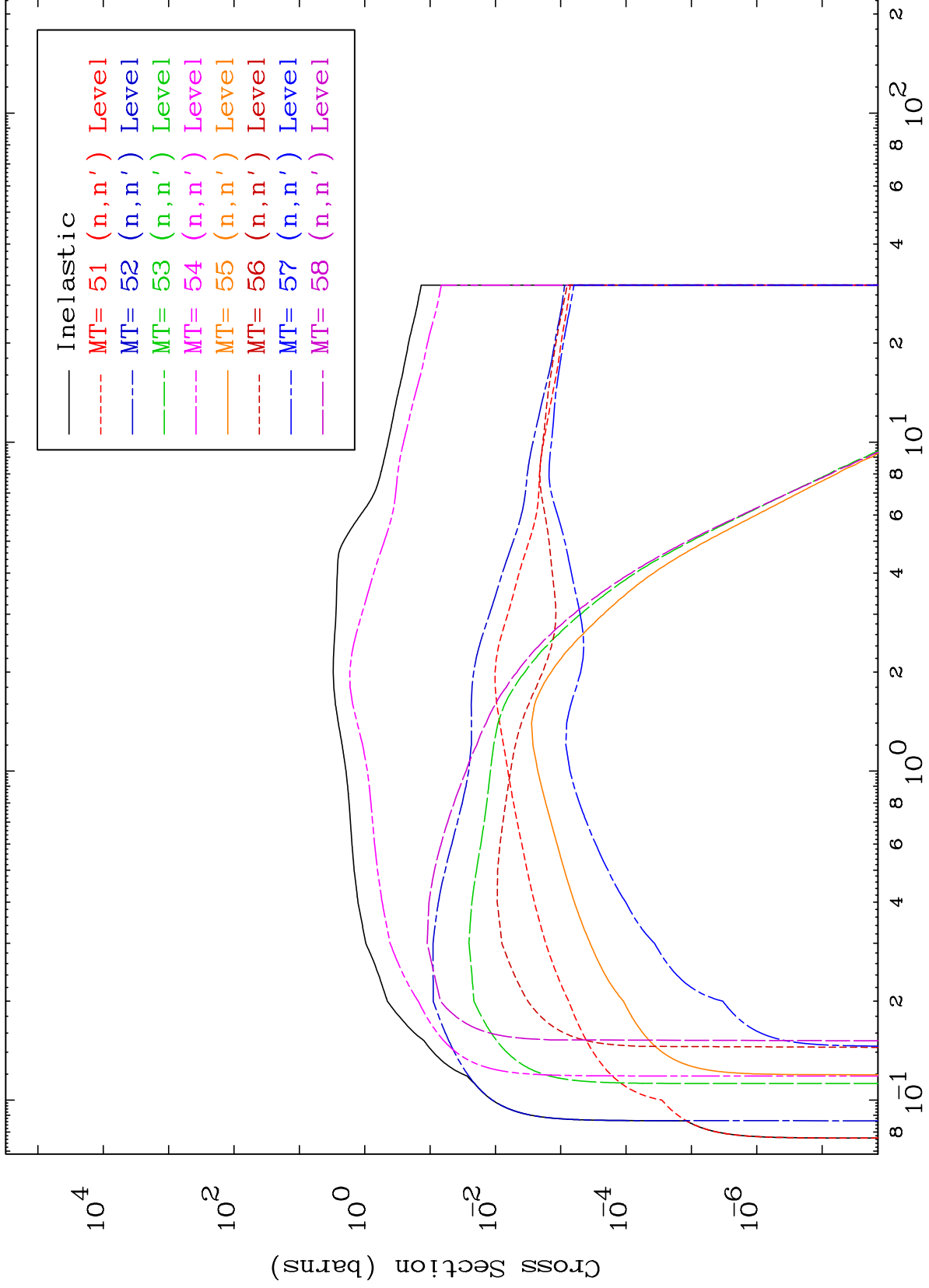


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(n,n') Levels

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293 Kelvin Cross Sections



9

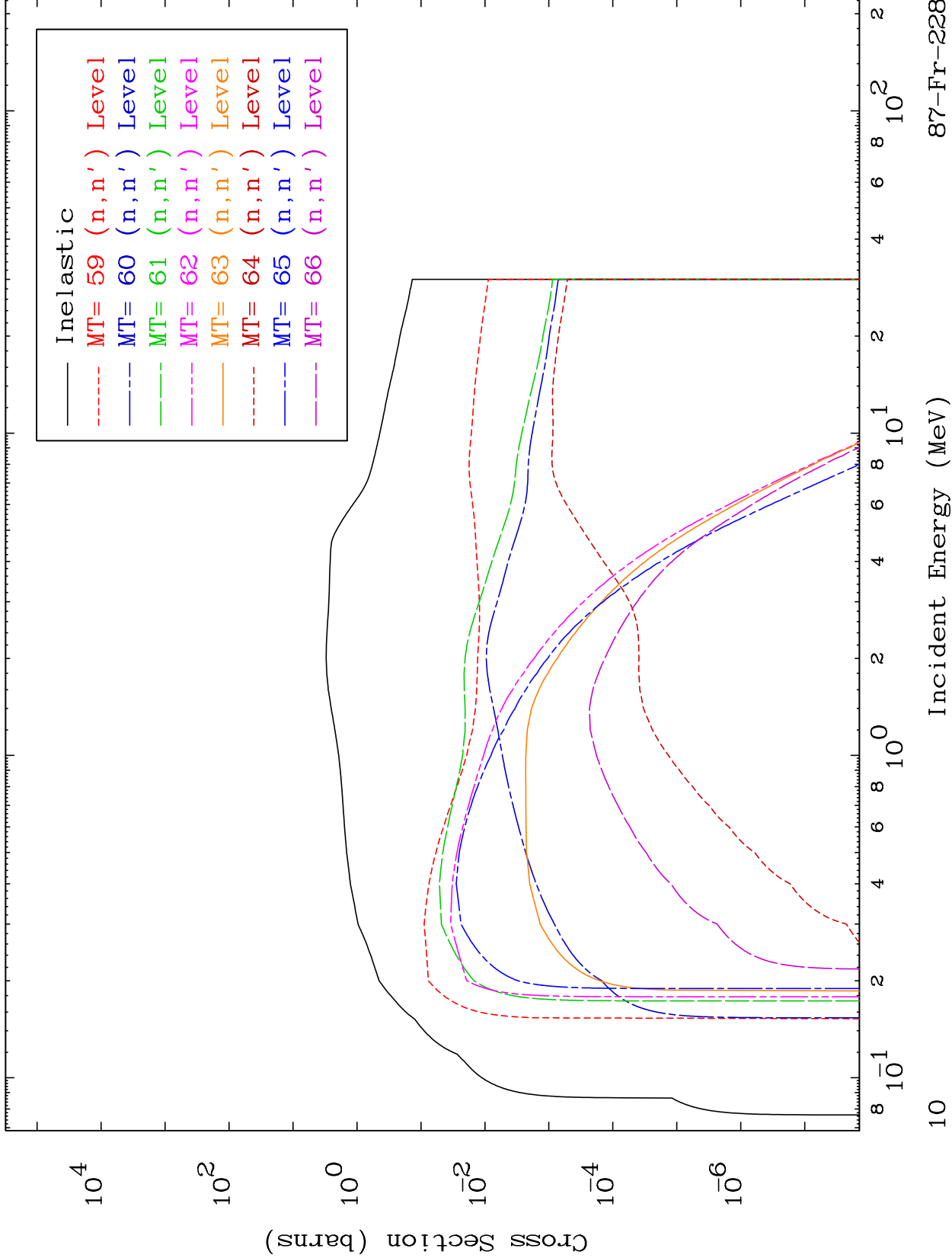
Incident Energy (MeV)

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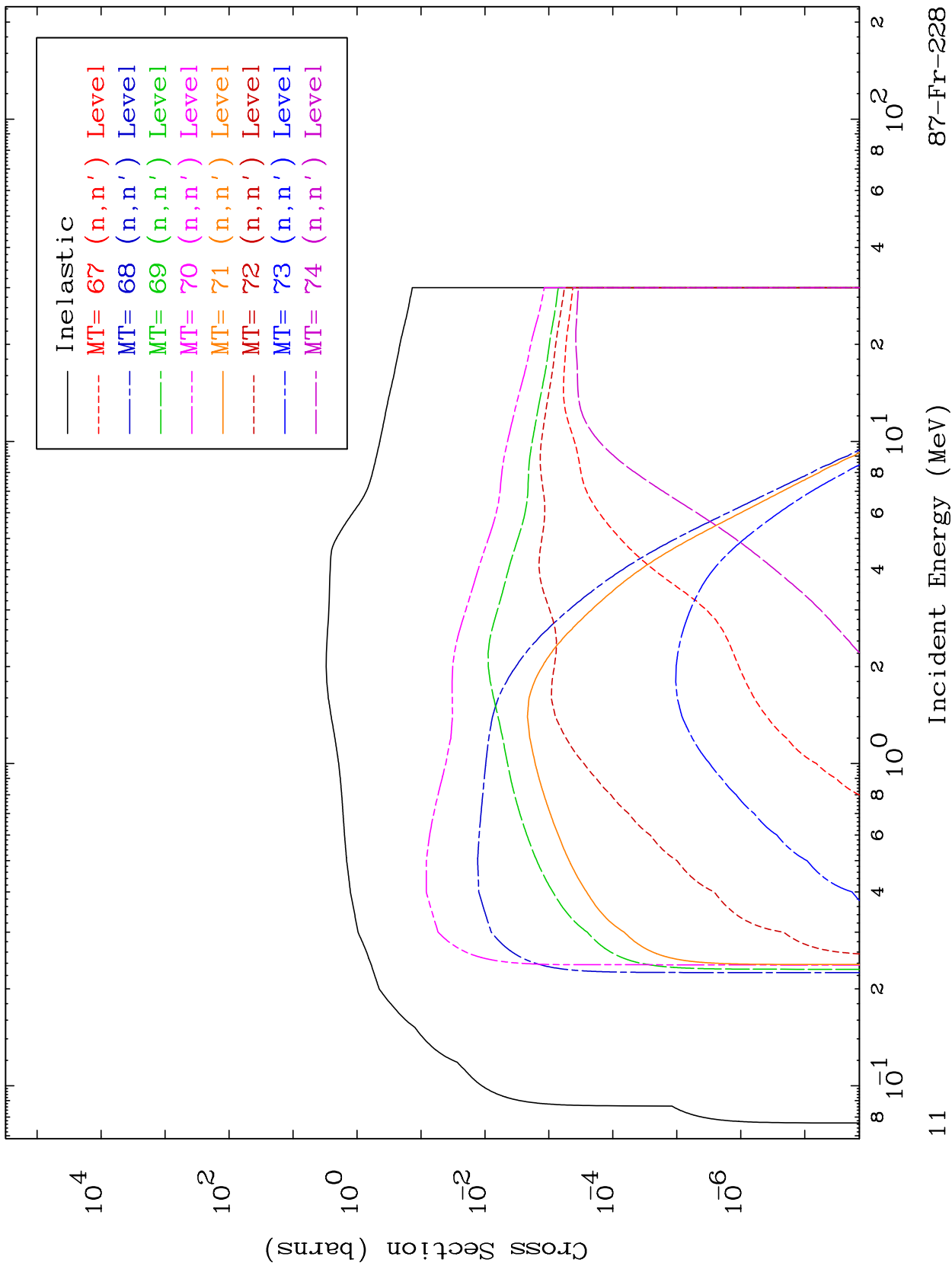
(n,n') Levels
293 Kelvin Cross Sections

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Incident Energy (MeV)

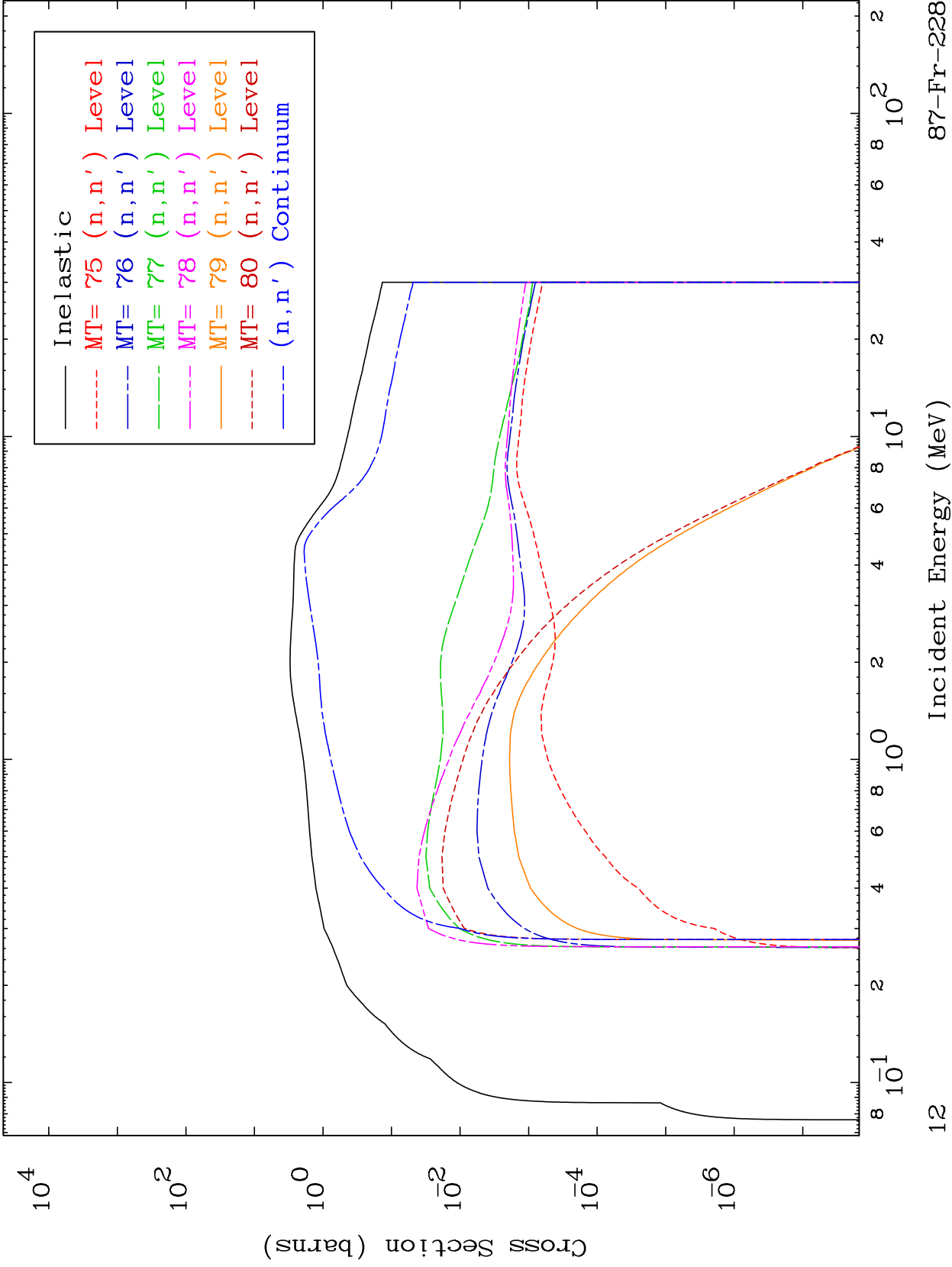


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(n,n') Levels

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293 Kelvin Cross Sections



12

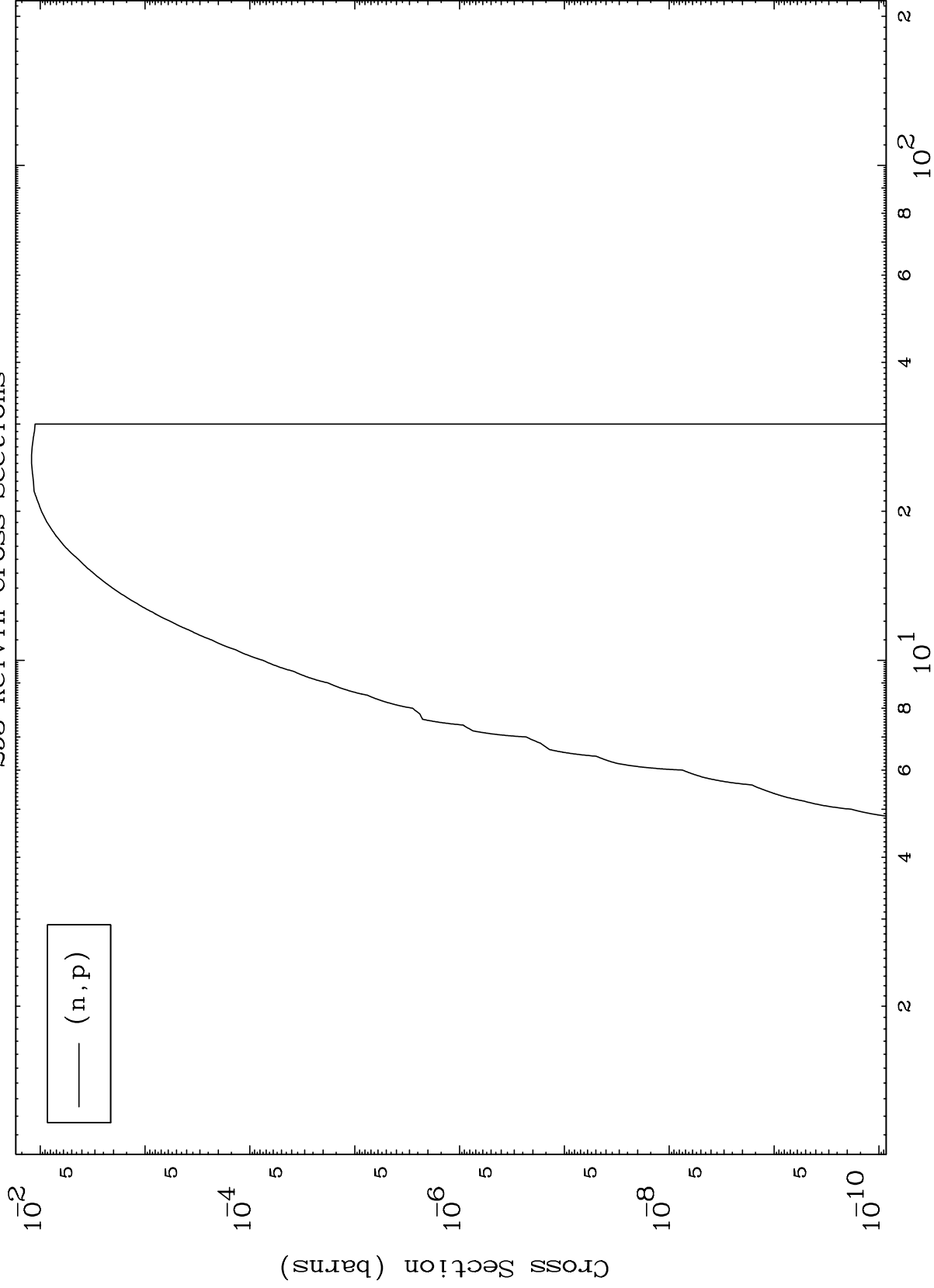
Incident Energy (MeV)

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MAT 8773

(n,p) Levels
293 Kelvin Cross Sections

87-Fr-228



13

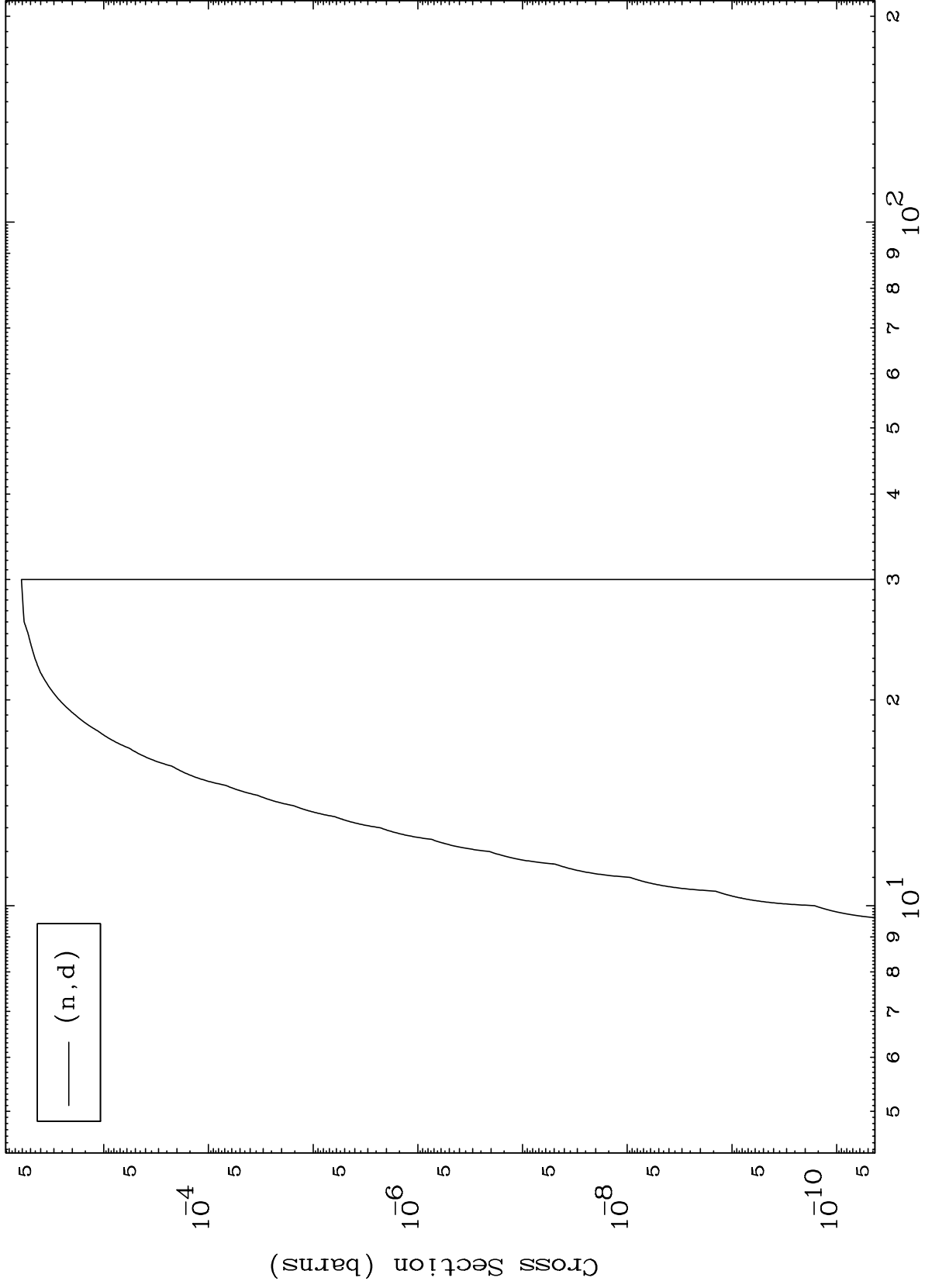
Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

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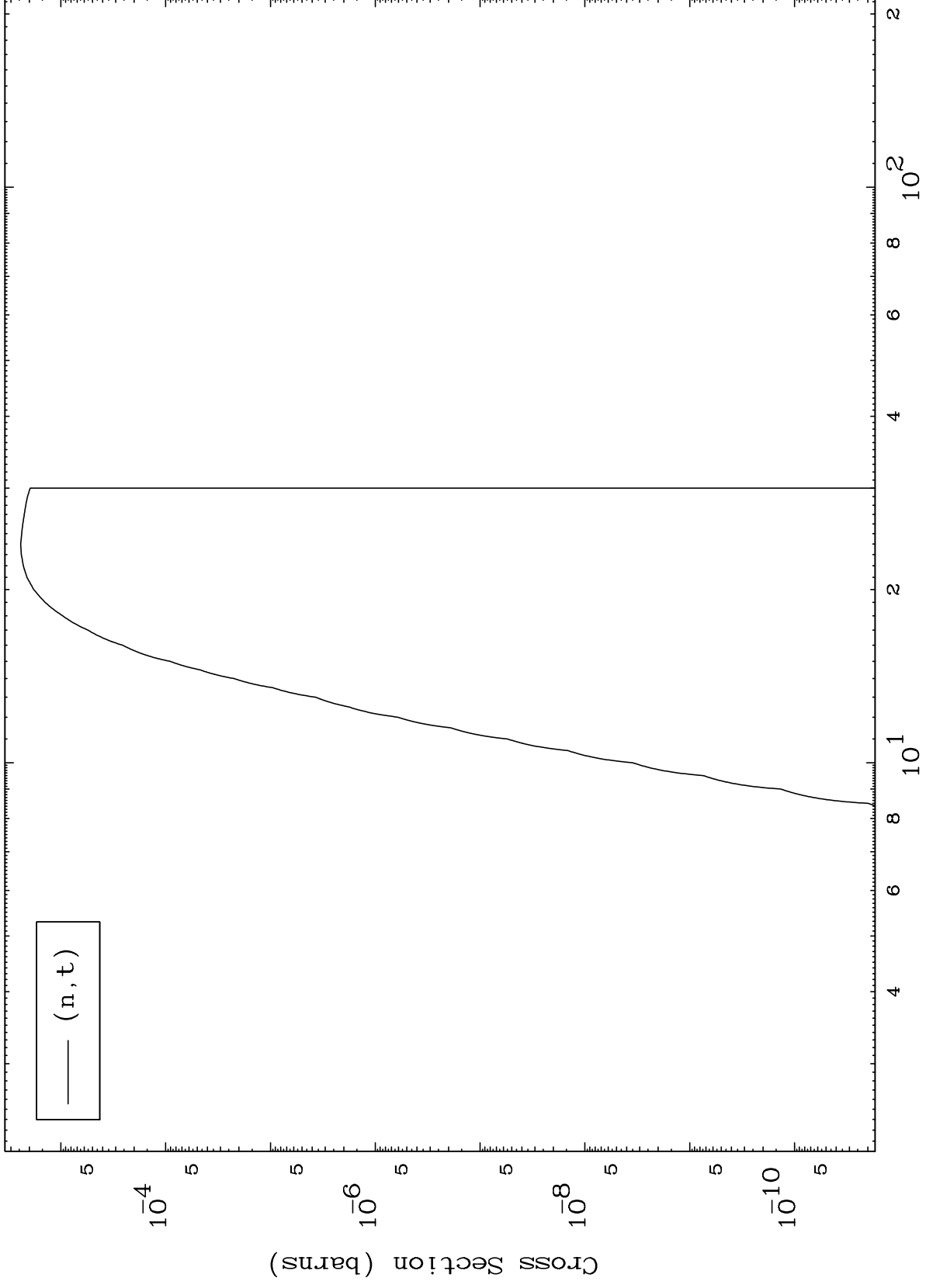
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

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15

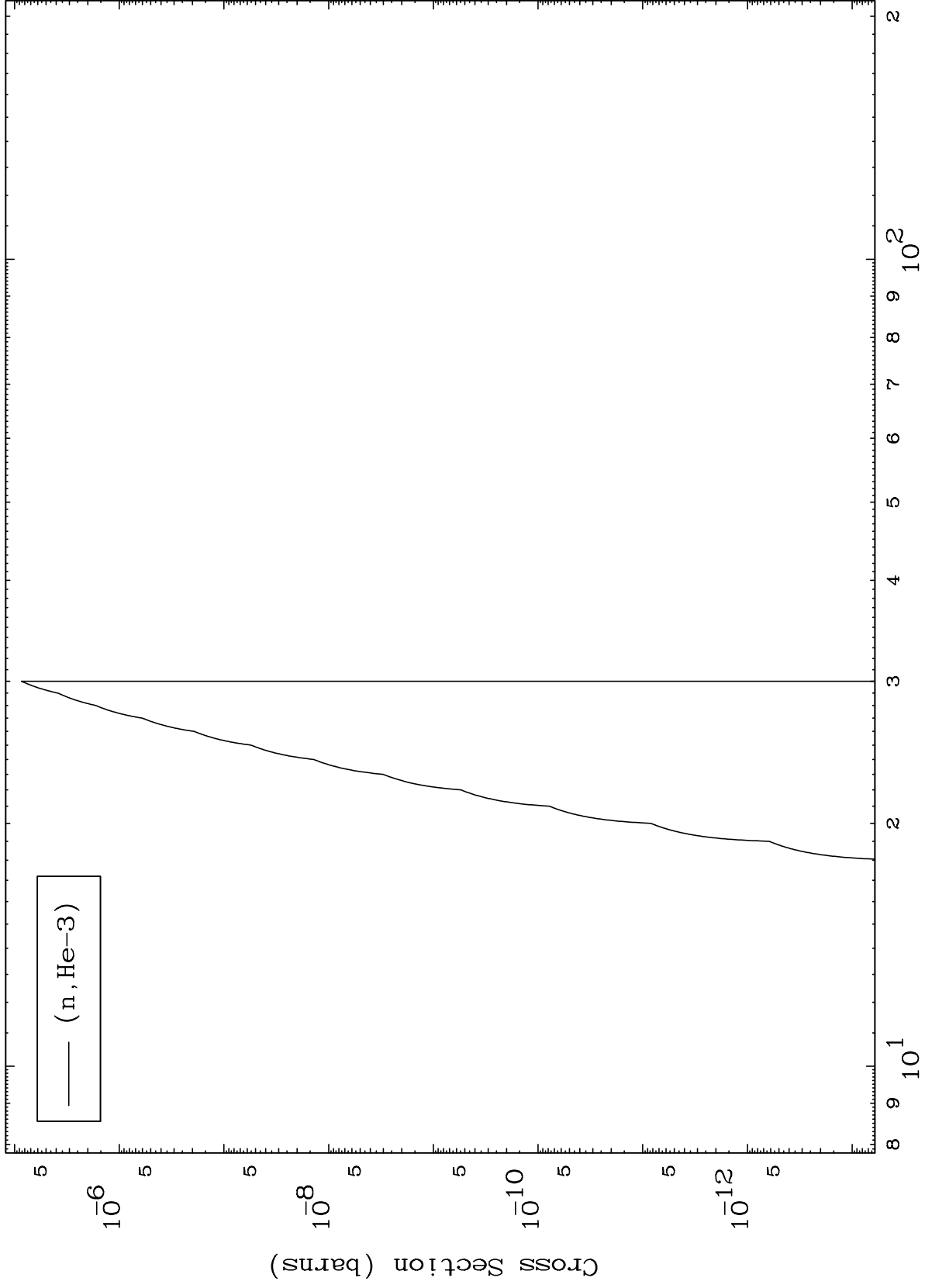
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

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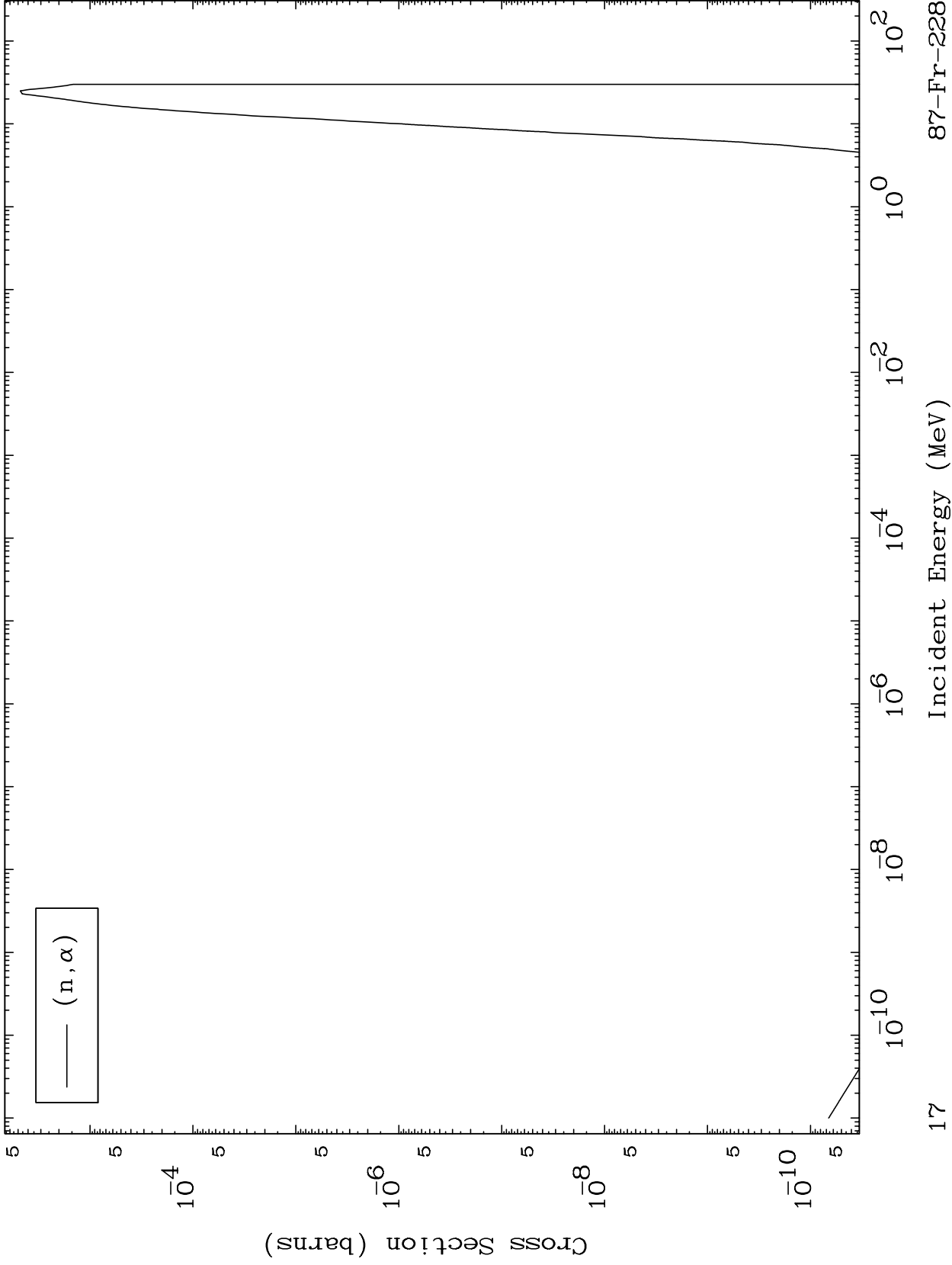
Incident Energy (MeV)

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(n, α) Levels
293 Kelvin Cross Sections

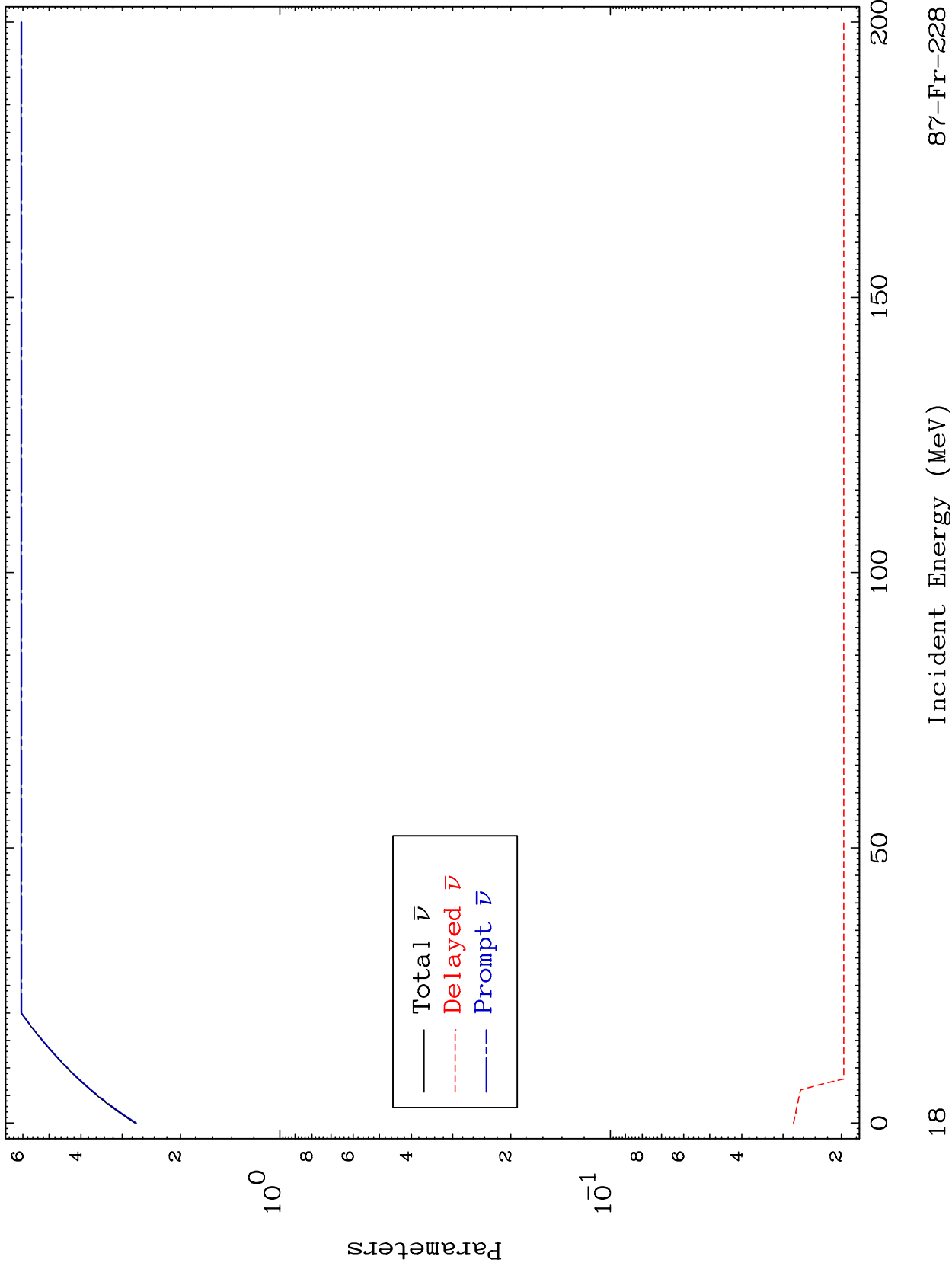
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Energy Release
Parameters

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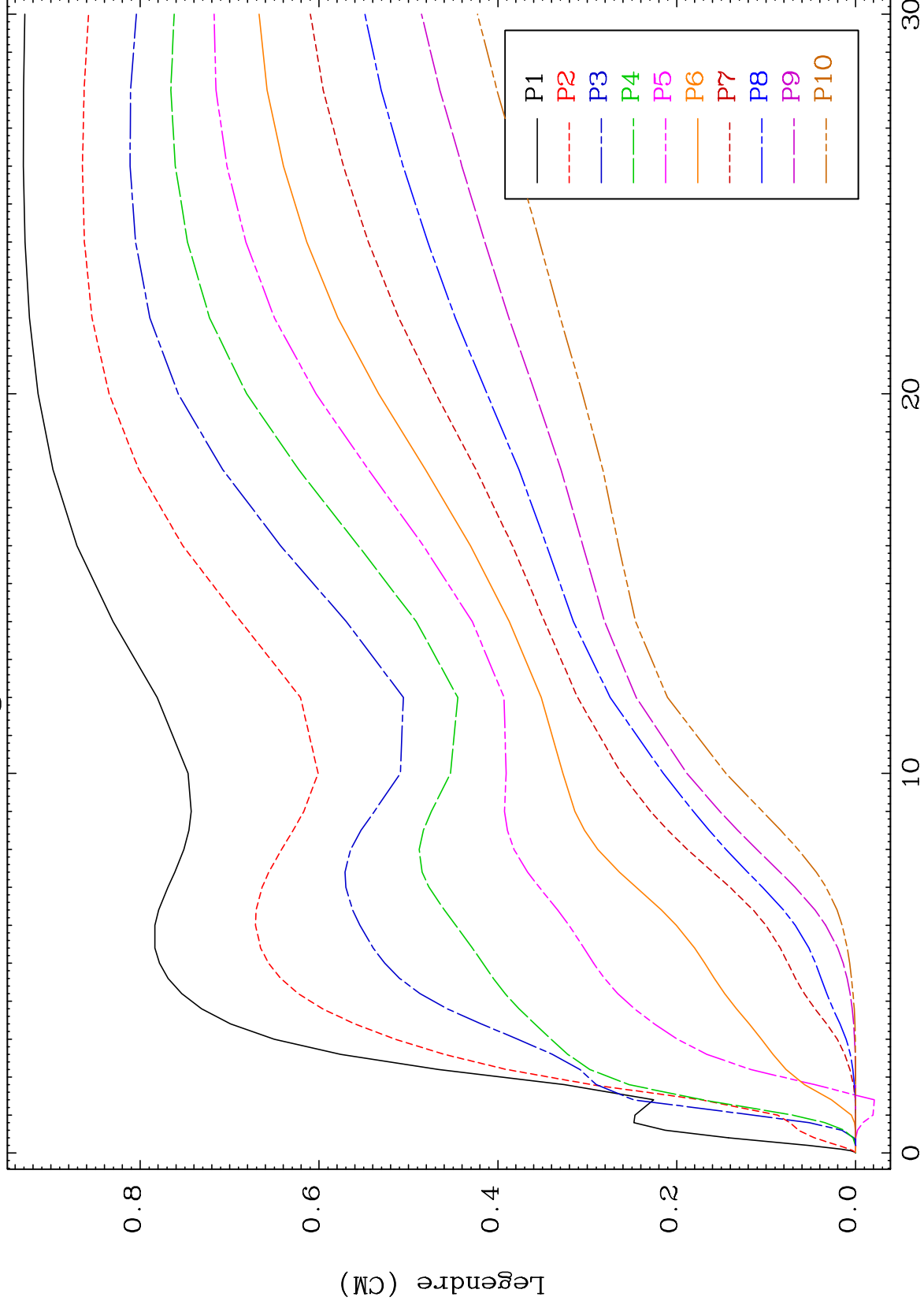
Incident Energy (MeV)

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Elastic Legendre Coefficients

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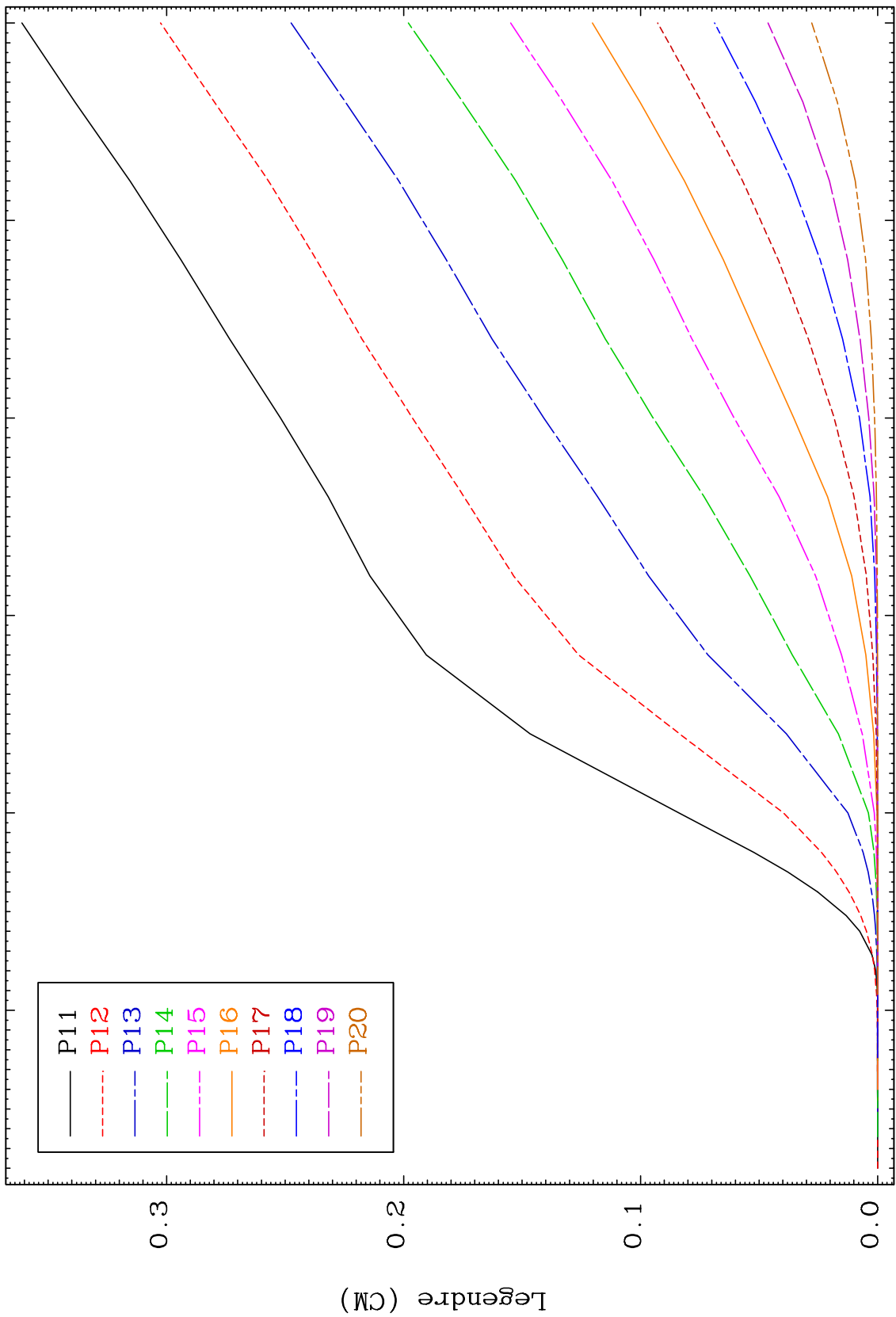
Incident Energy (MeV)

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Elastic Legendre Coefficients

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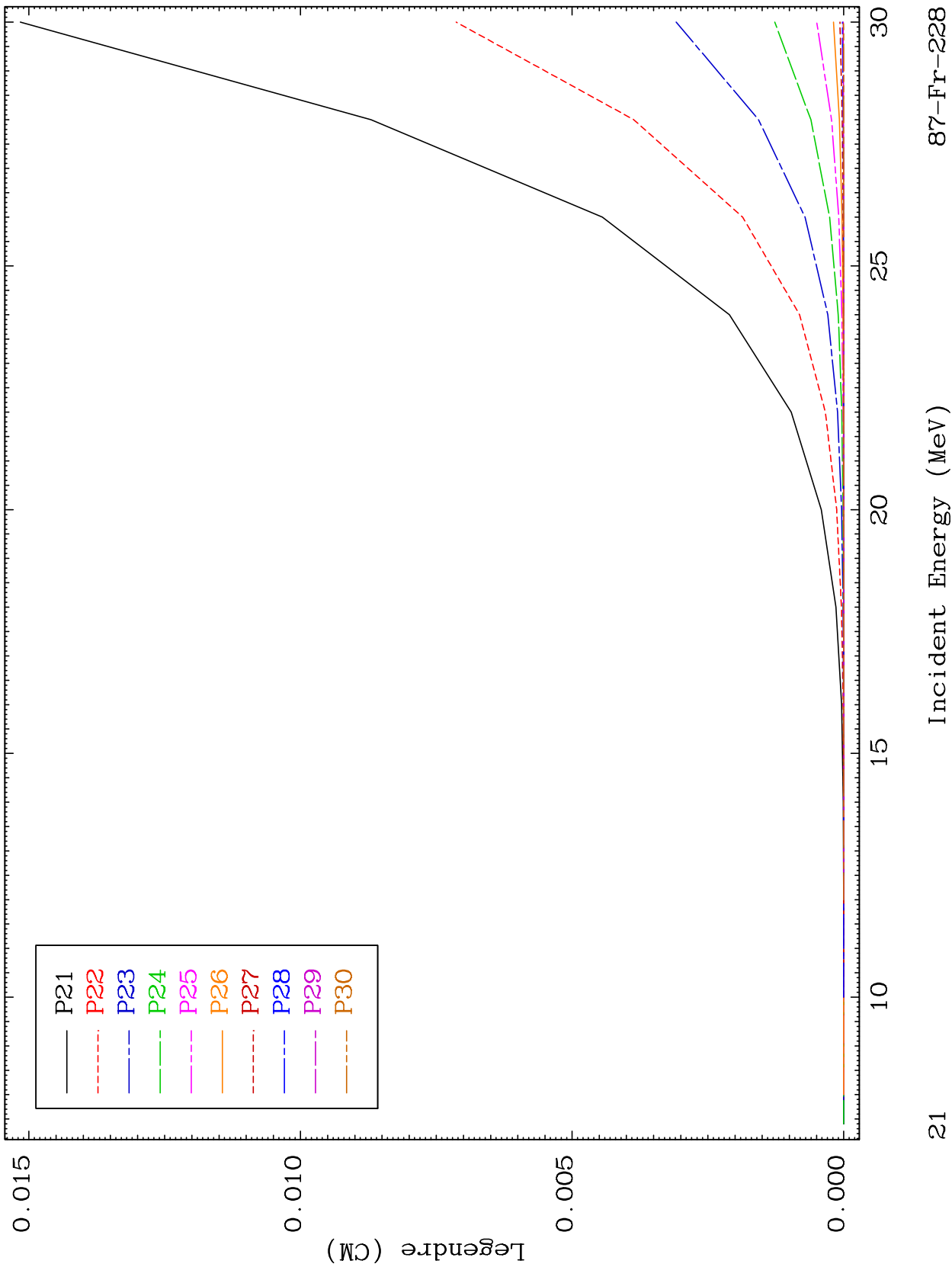
Incident Energy (MeV)

87-Fr-228

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Elastic Legendre Coefficients

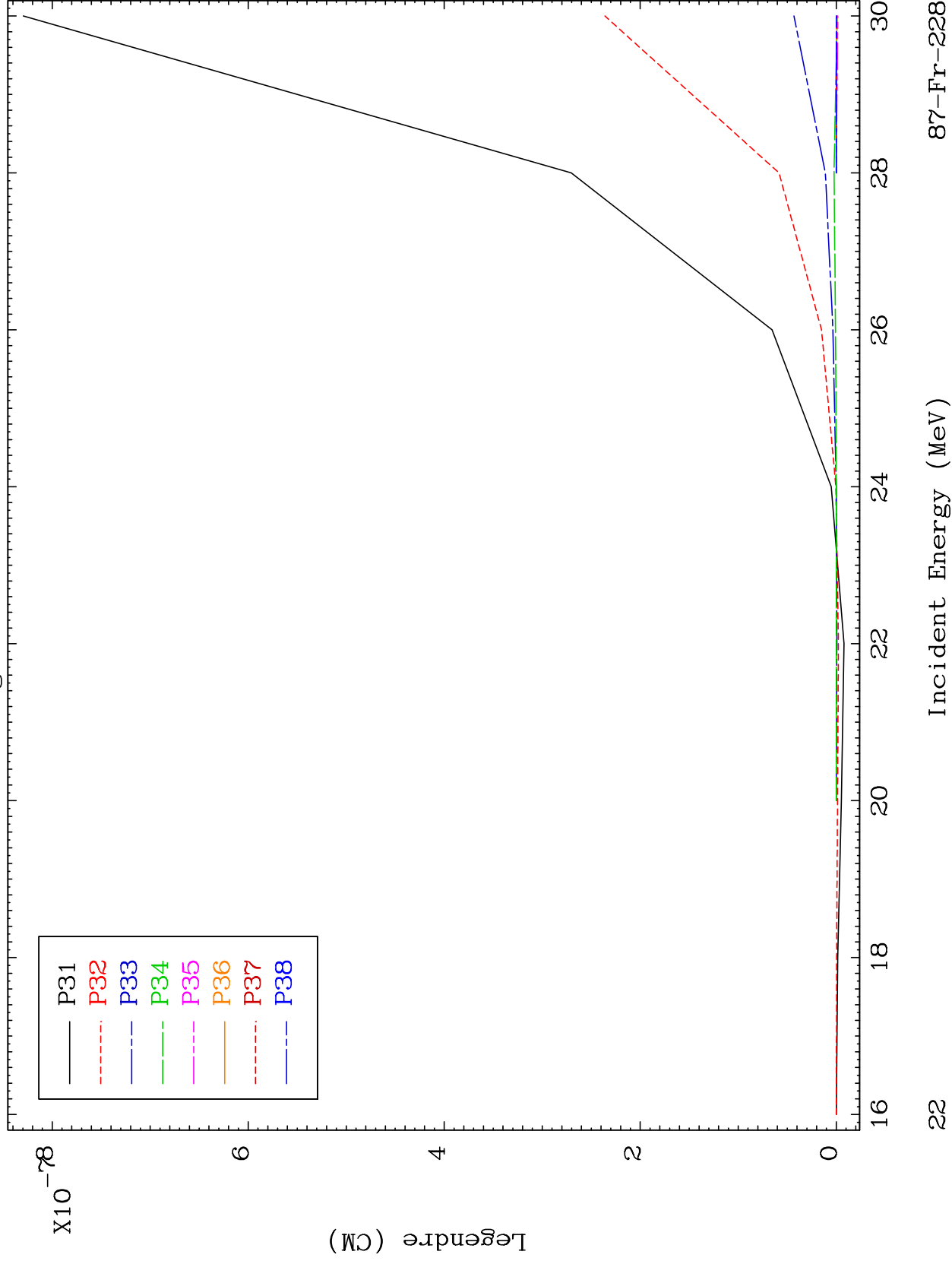
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Elastic
Legendre Coefficients

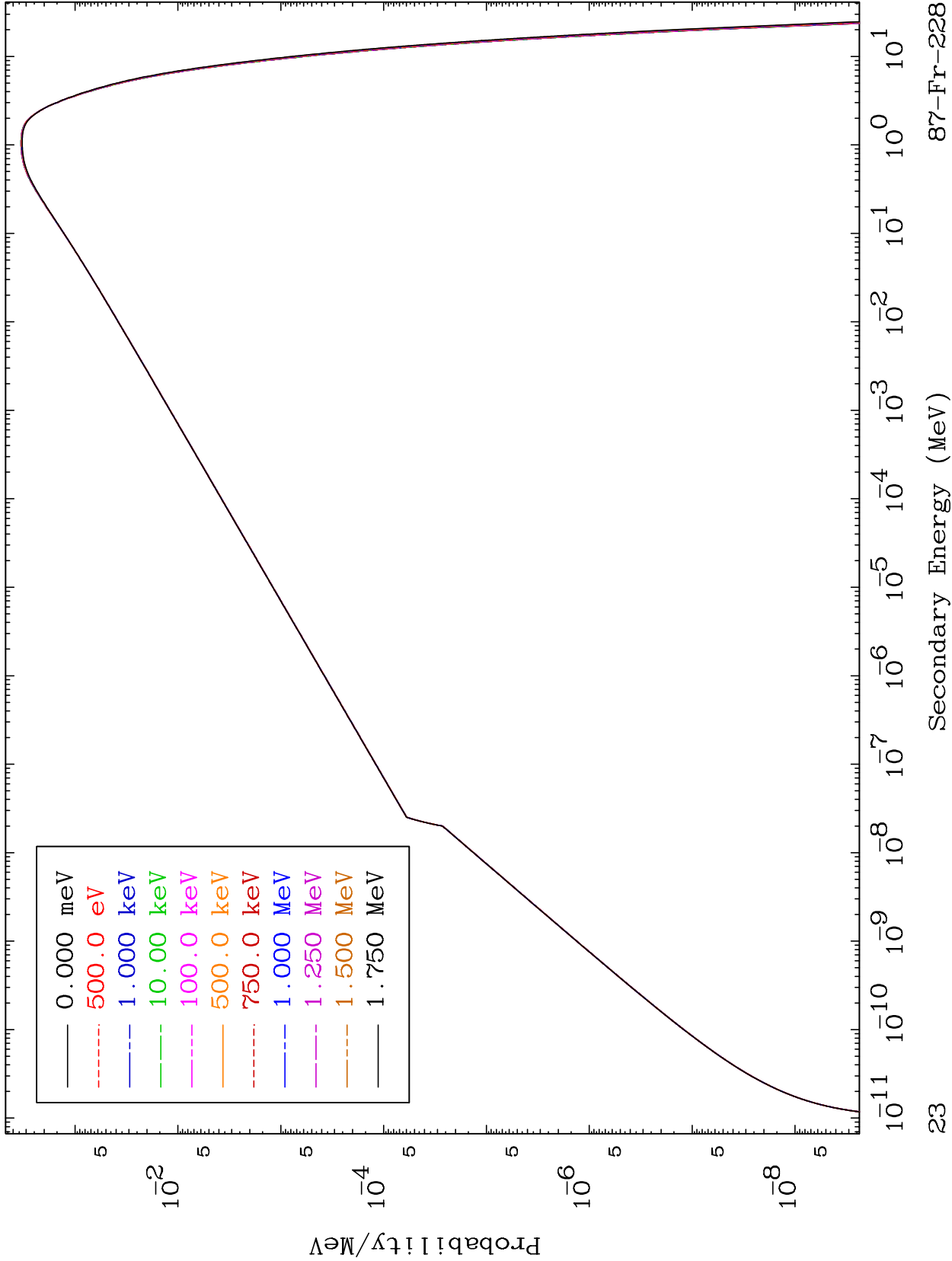
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Fission Energy Distributions

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- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

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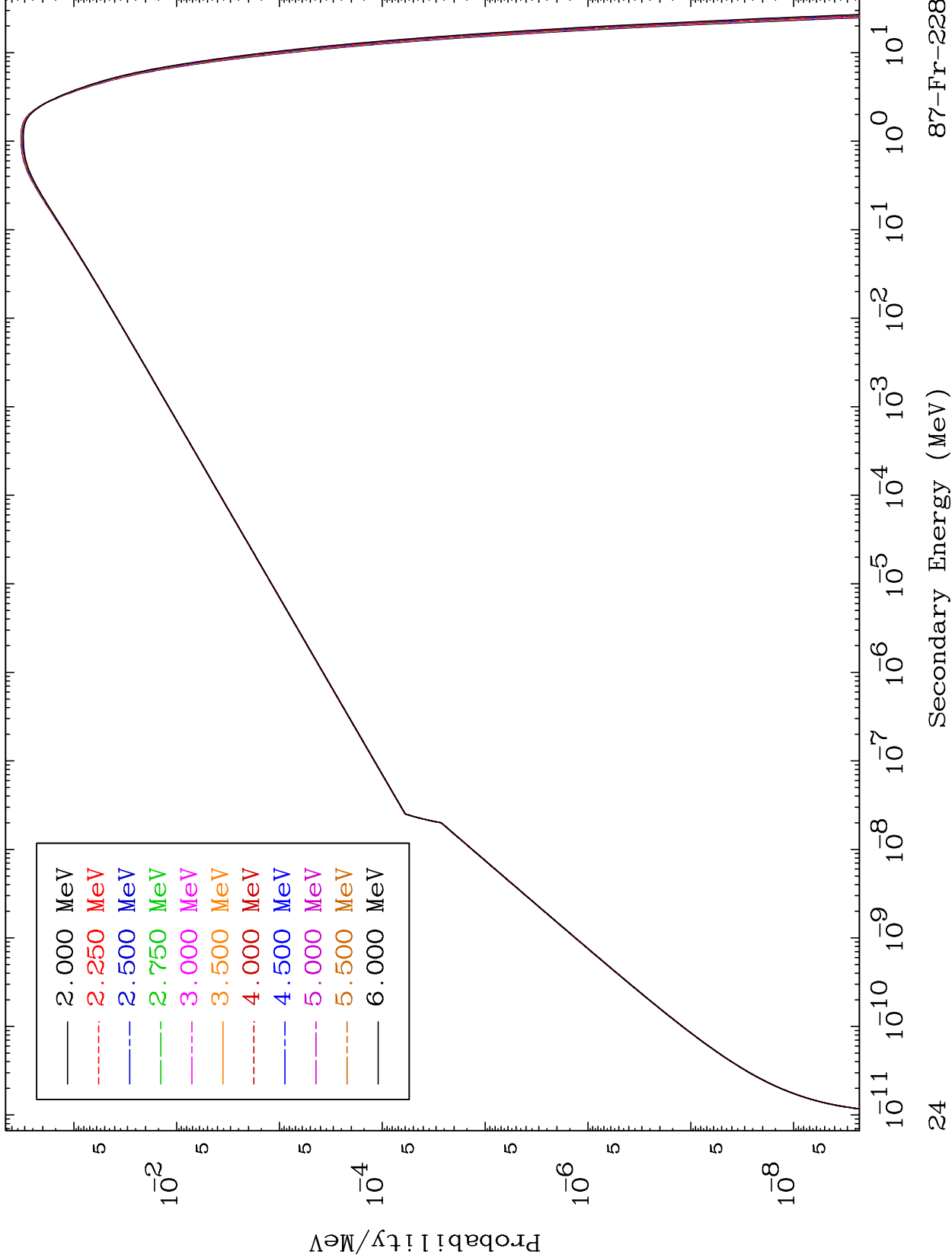
Secondary Energy (MeV)

23

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Fission Energy Distributions

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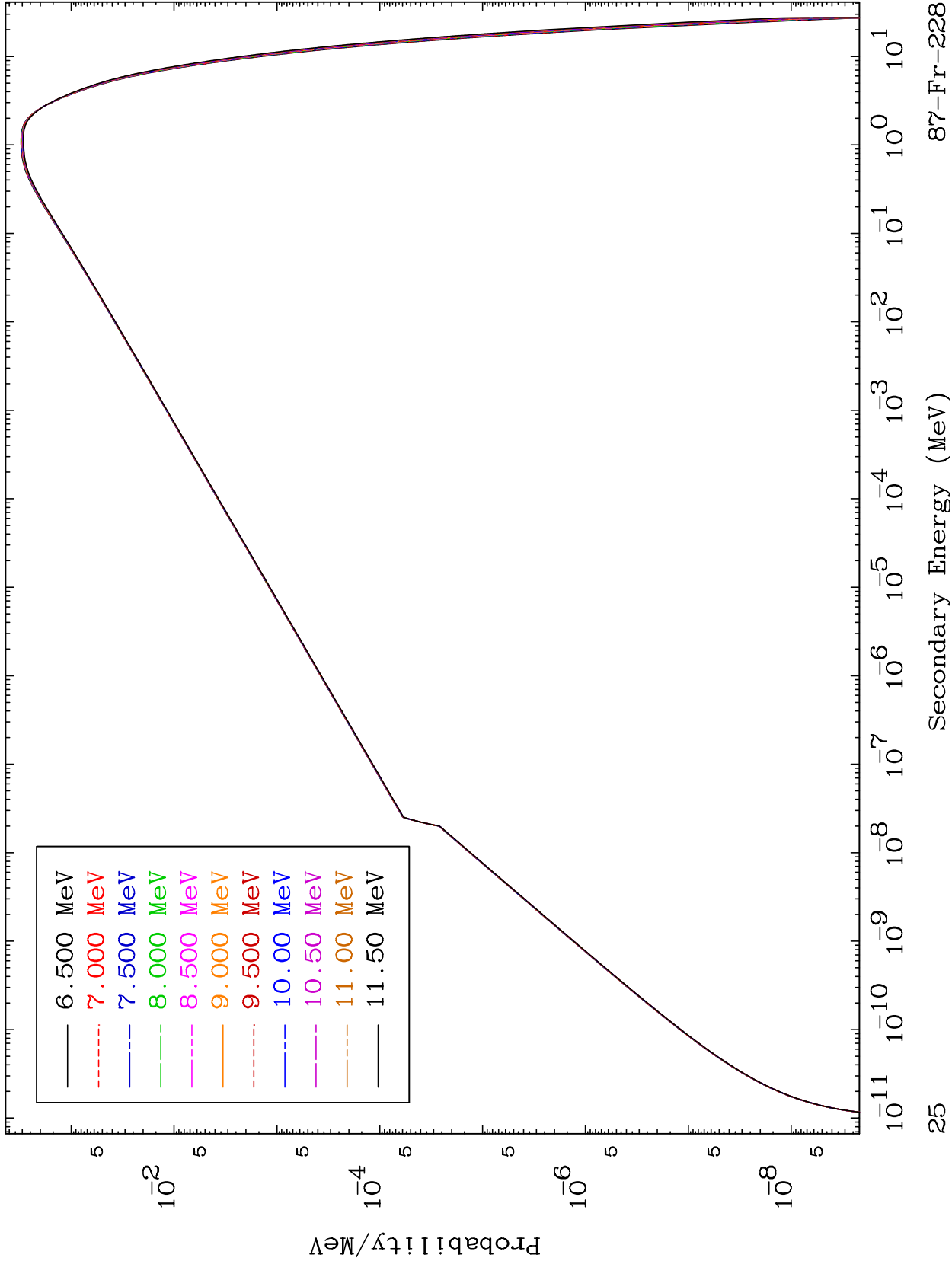


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Fission Energy Distributions

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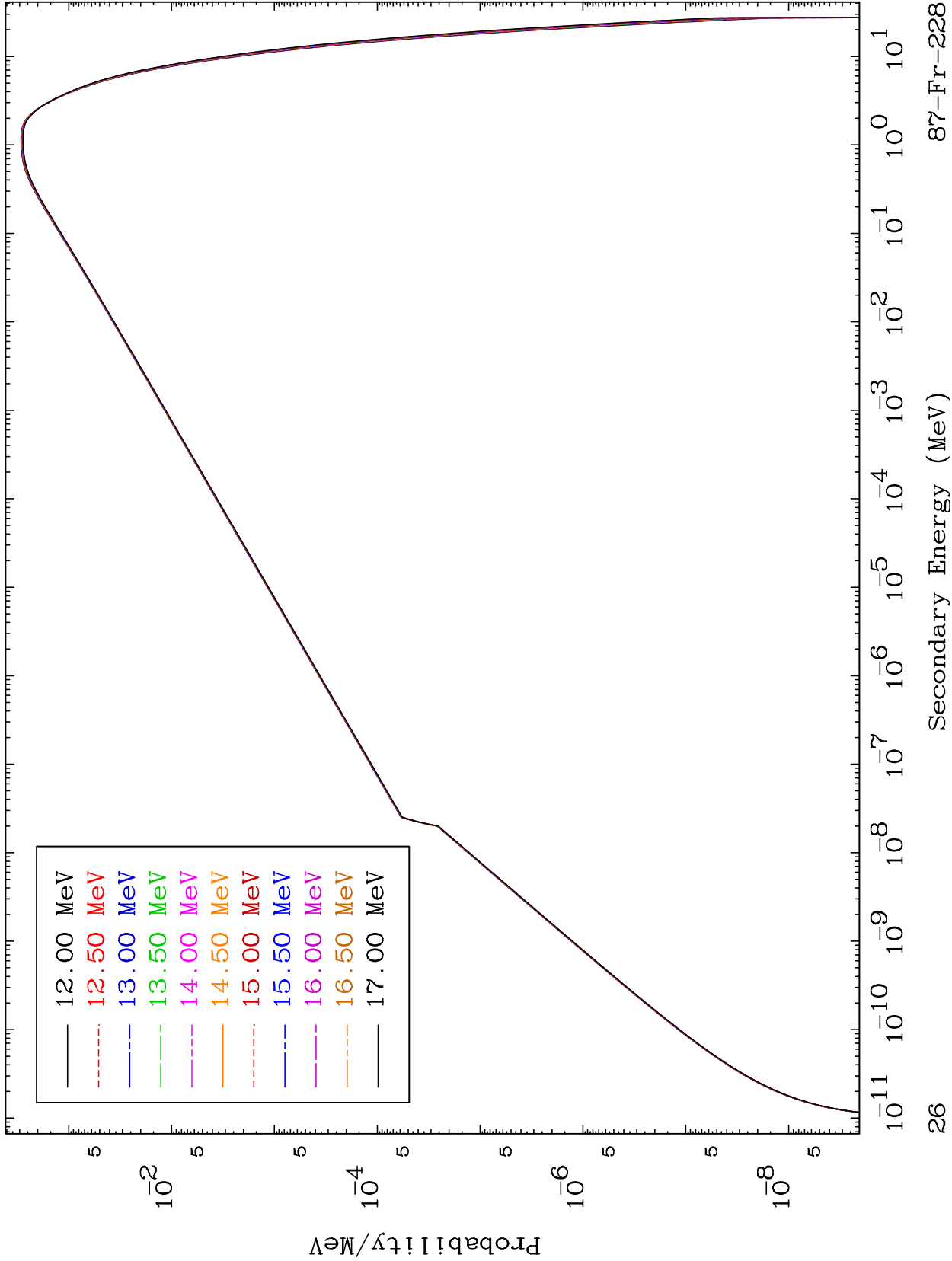


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Secondary Energy (MeV)

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Fission Energy Distributions



Fission Energy Distributions

