

Program EVALPLOT
(Version 2021-1)

by

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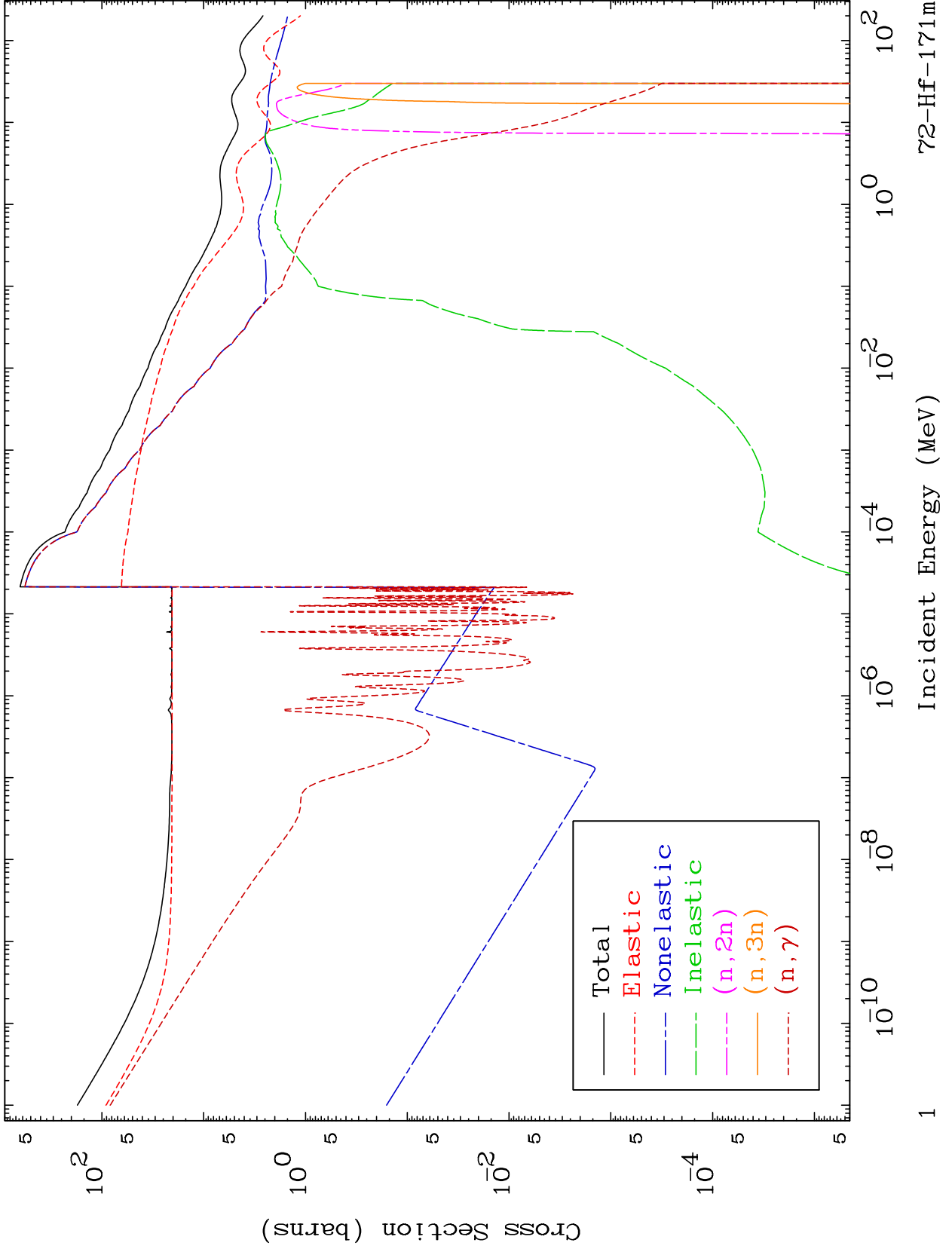
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Press Mouse Button to Start

MAT 7217

Neutron Major
293 Kelvin Cross Sections

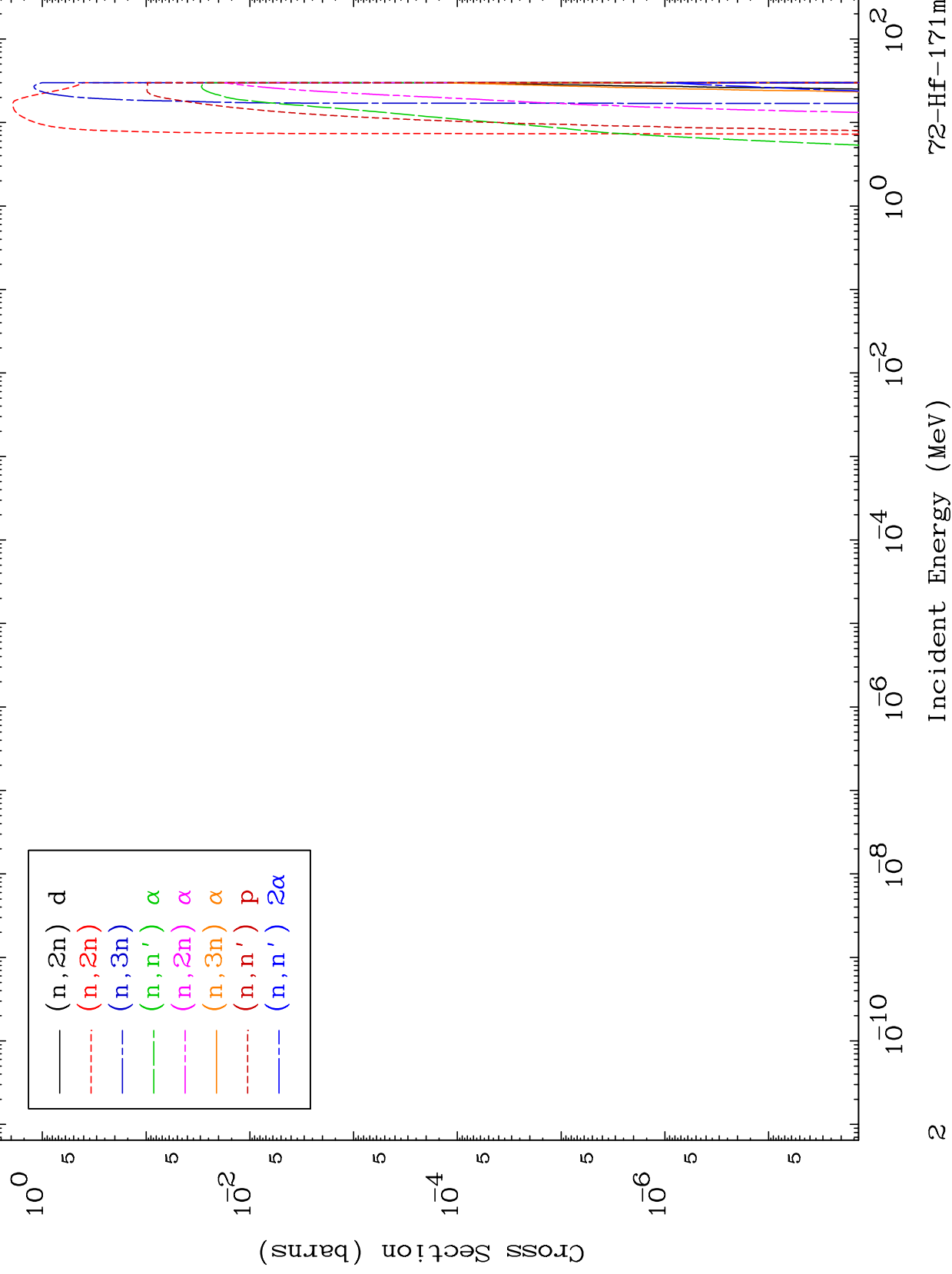
72-Hf-171m



MAT 7217

Neutron Absorption
293 Kelvin Cross Sections

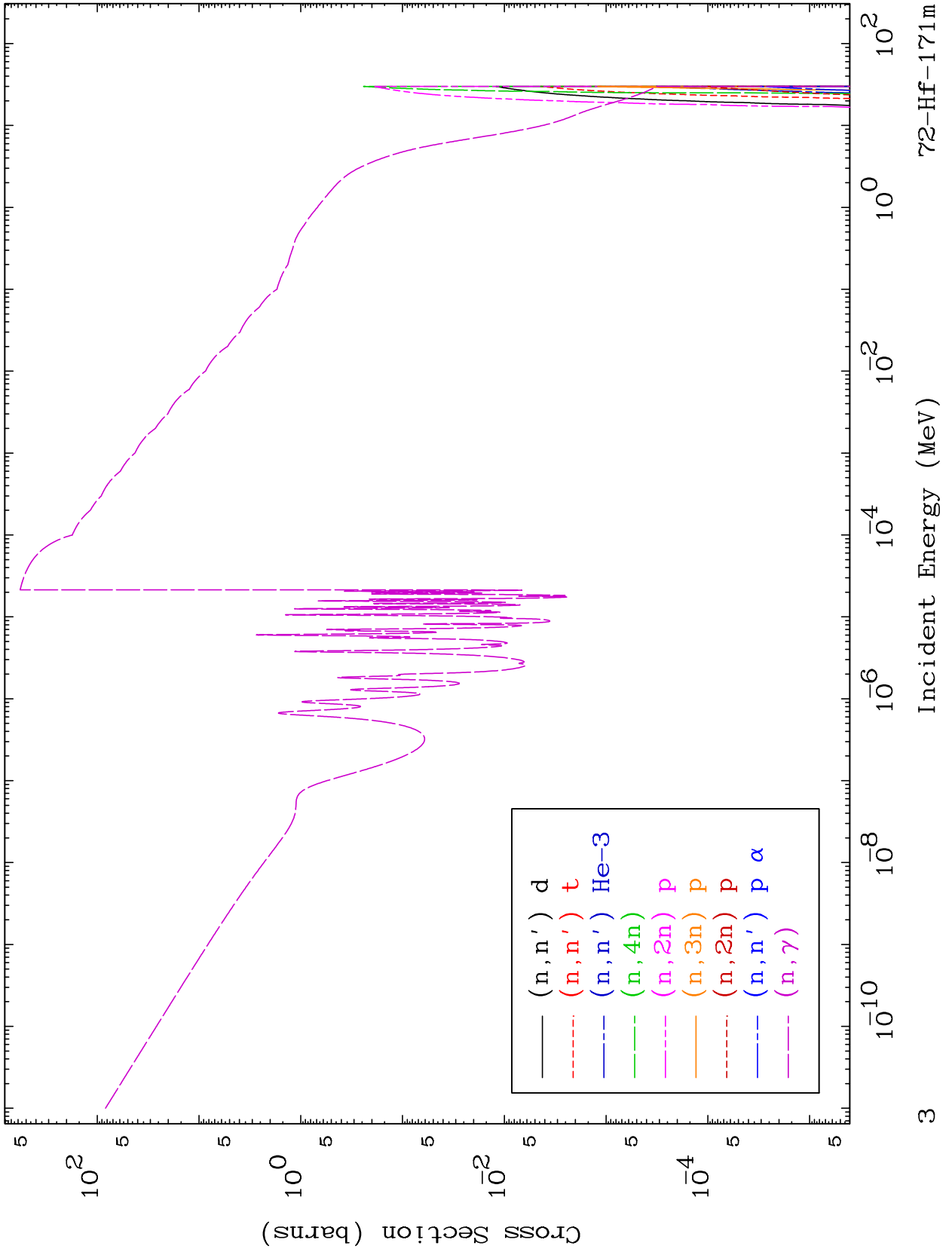
72-Hf-171m



MAT 7217

Neutron Absorption
293 Kelvin Cross Sections

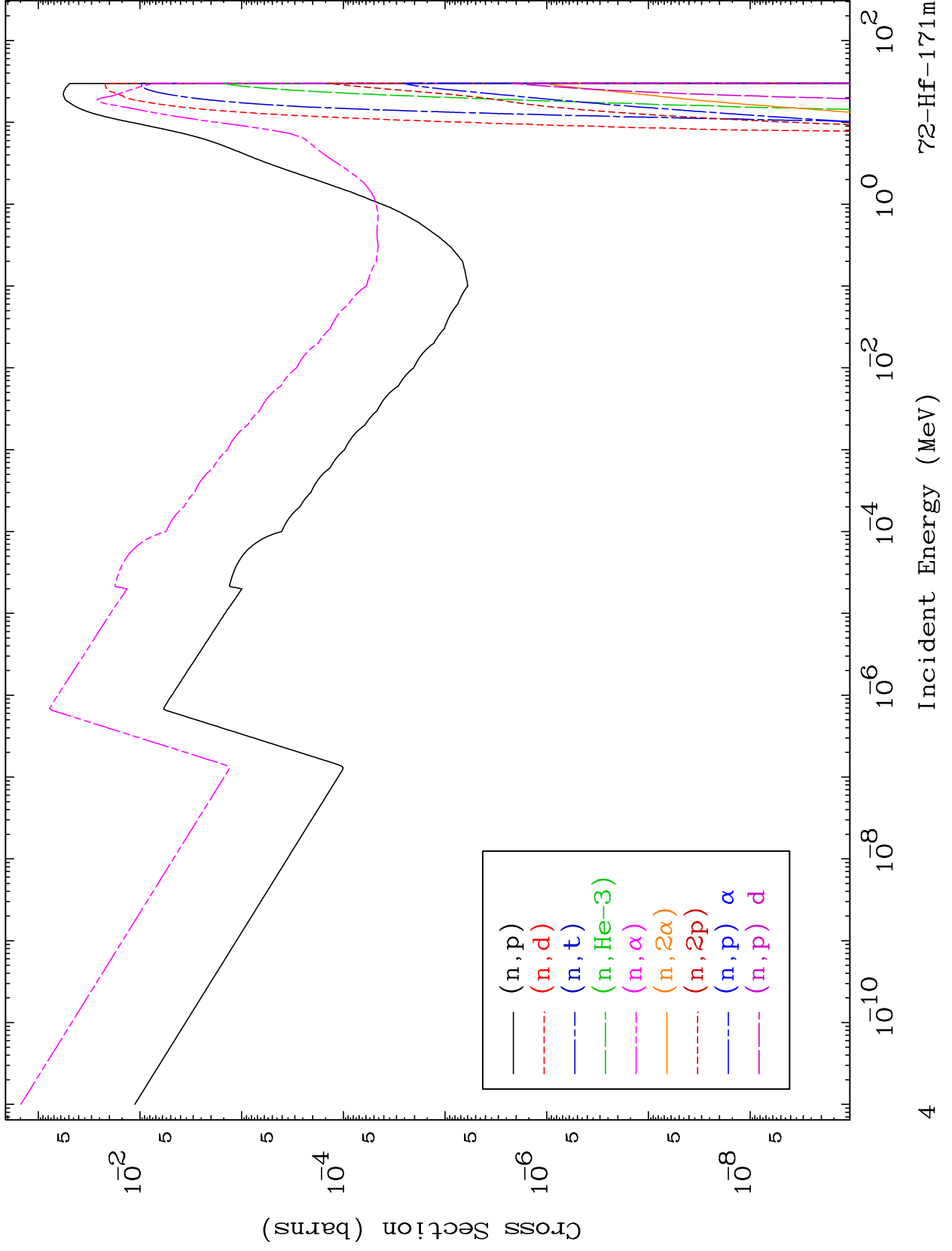
72-Hf-171m



MAT 7217

Neutron Absorption
293 Kelvin Cross Sections

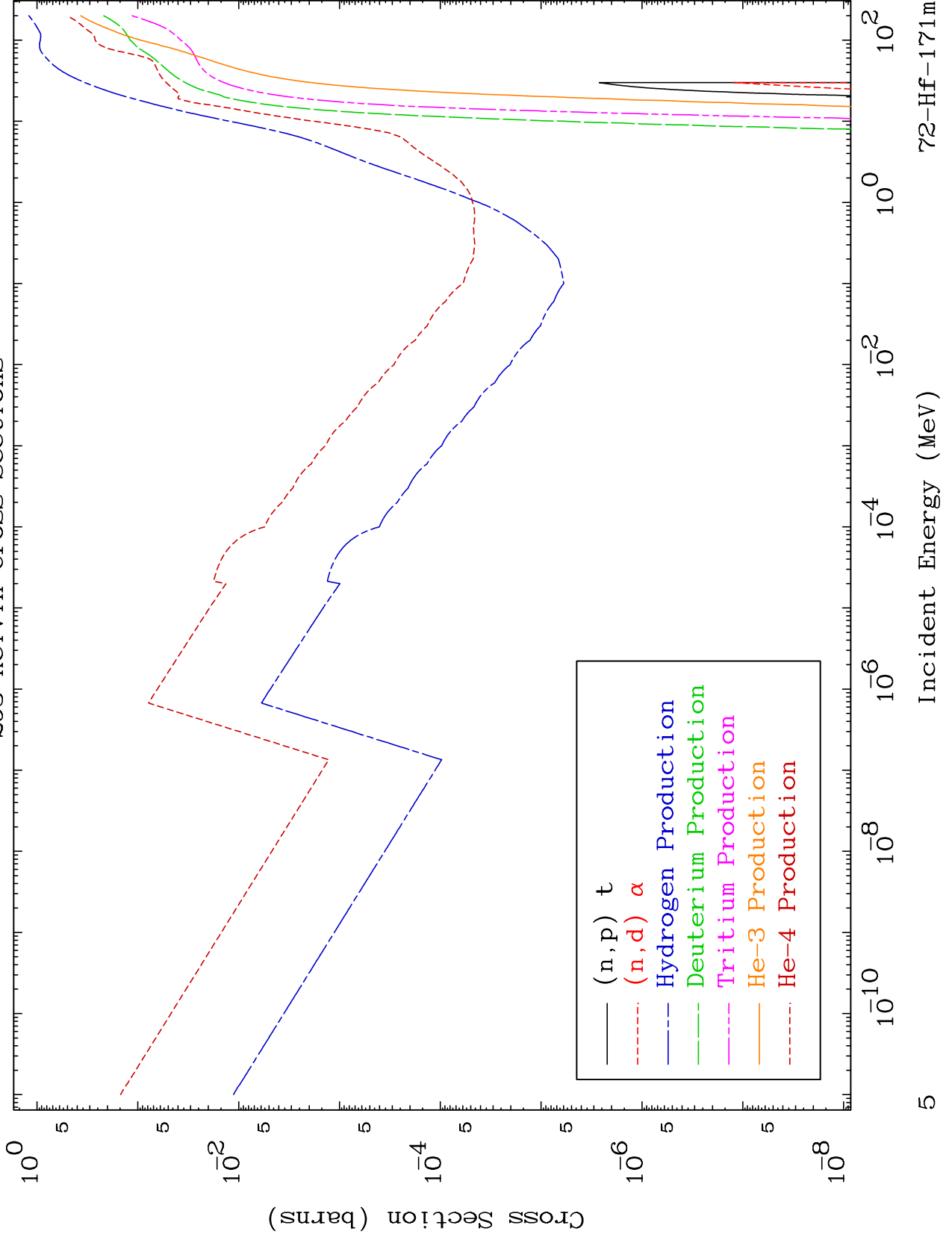
72-Hf-171m



MAT 7217

Neutron Absorption
293 Kelvin Cross Sections

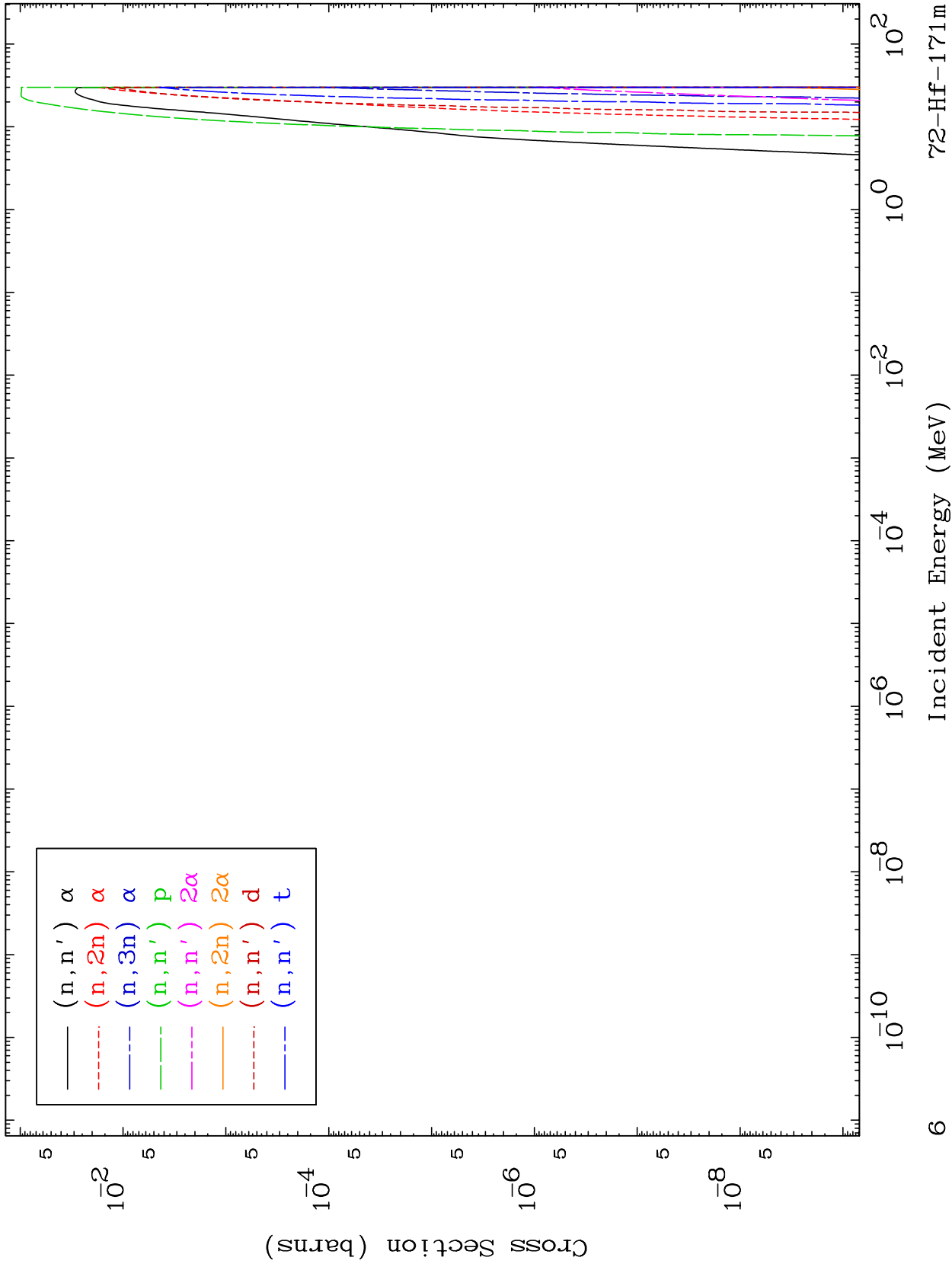
72-Hf-171m



MAT 7217

Charged Particle
293 Kelvin Cross Sections

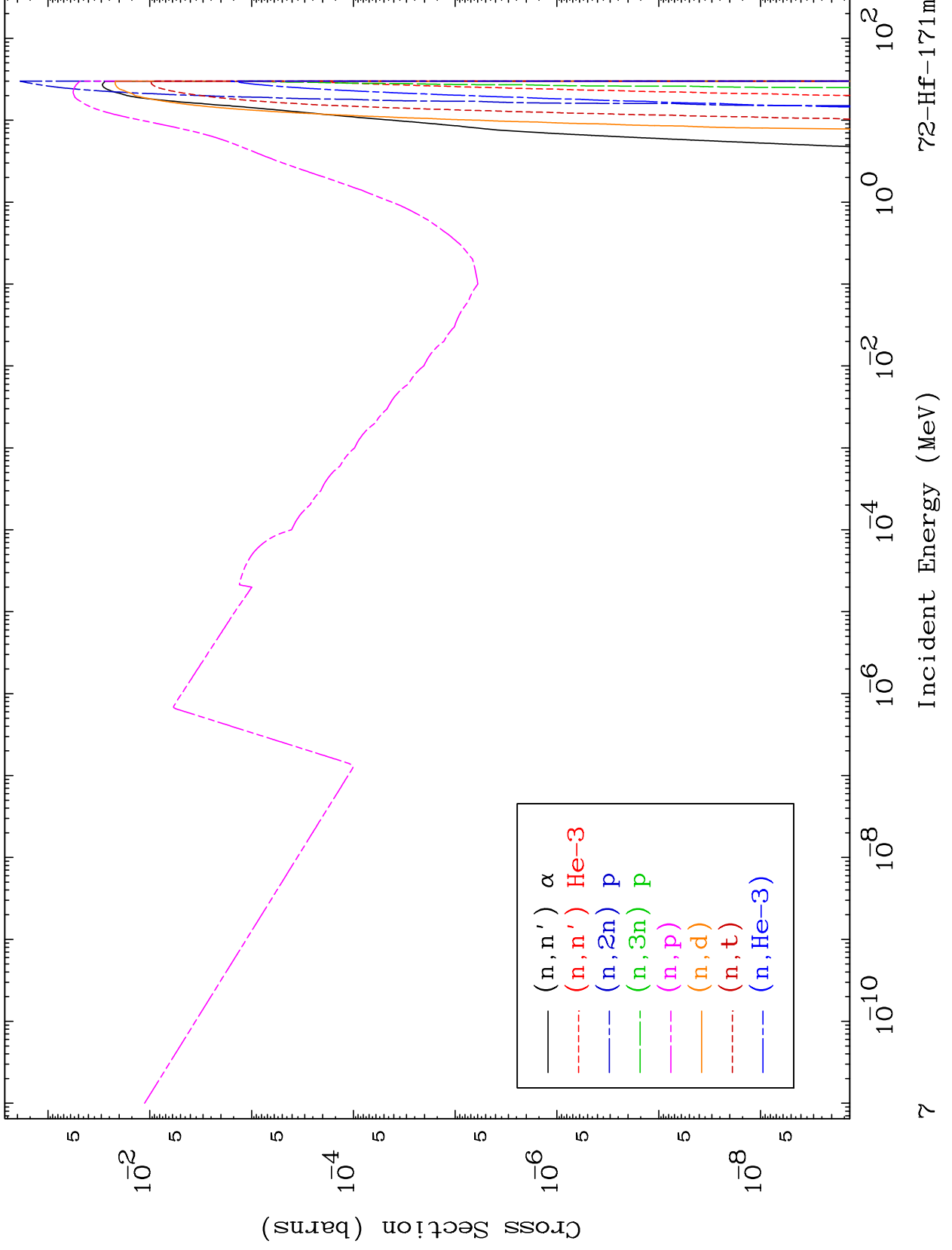
72-Hf-171m



MAT 7217

Charged Particle
293 Kelvin Cross Sections

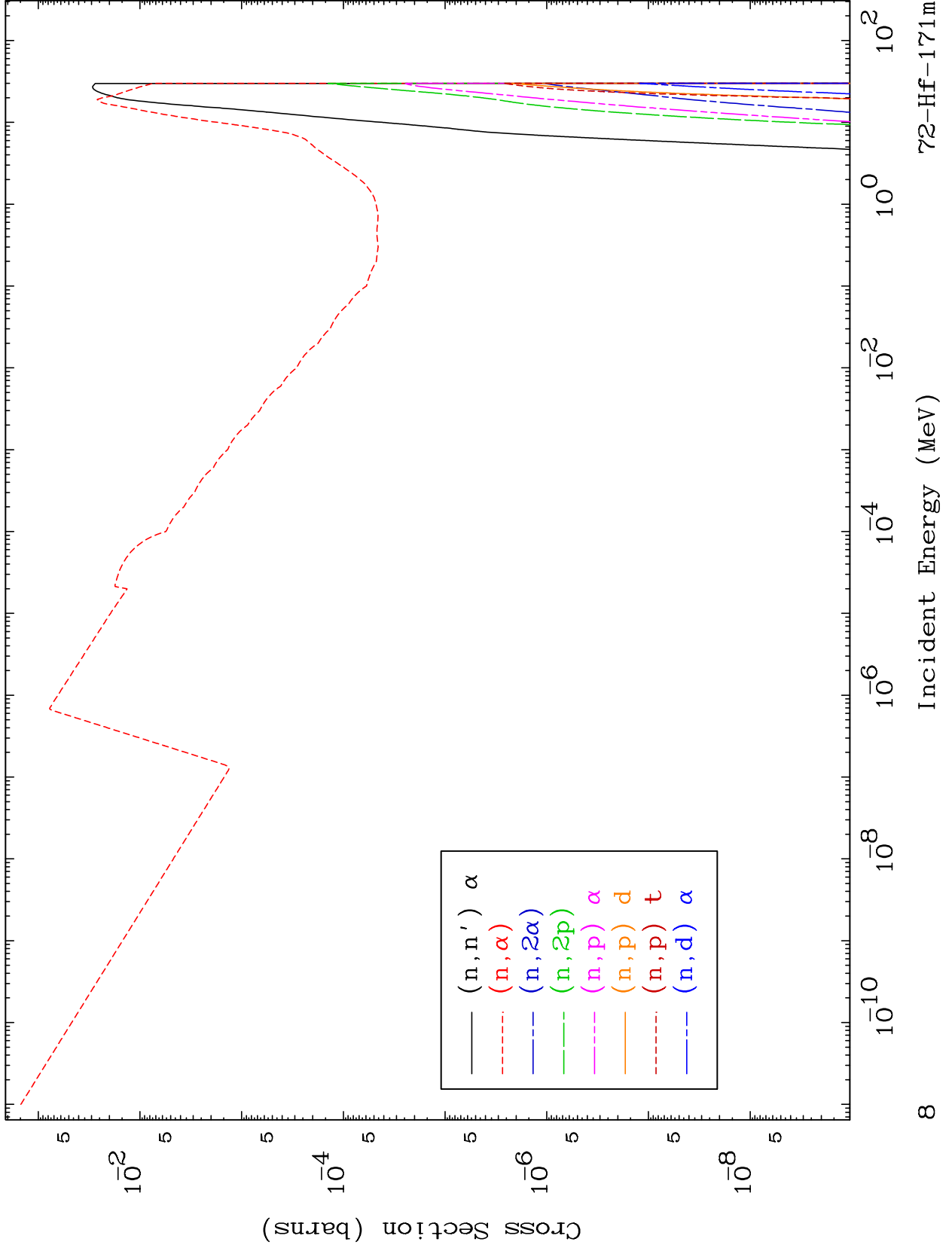
72-Hf-171m



MAT 7217

Charged Particle
293 Kelvin Cross Sections

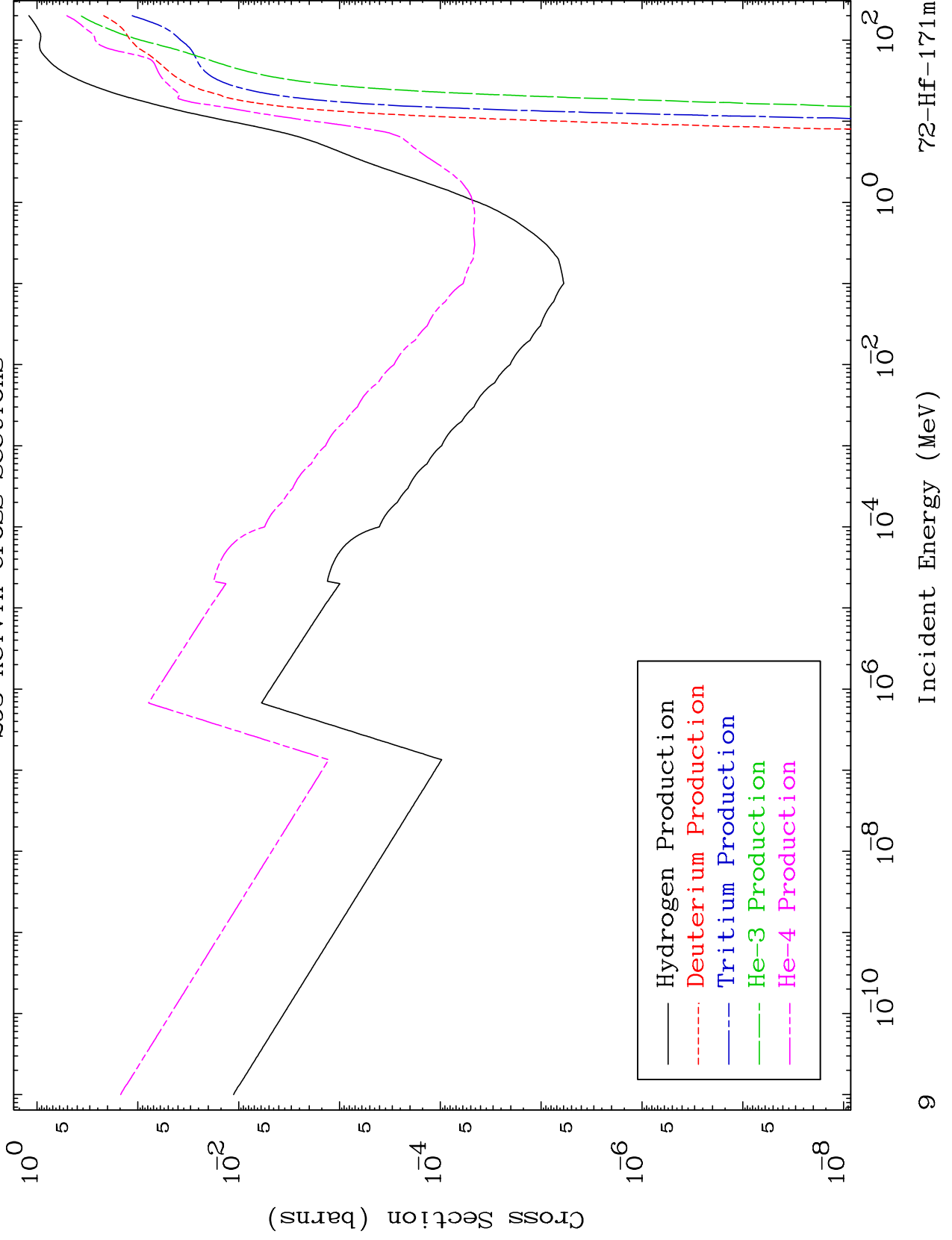
72-Hf-171m



MAT 7217

Particle Production
293 Kelvin Cross Sections

72-Hf-171m

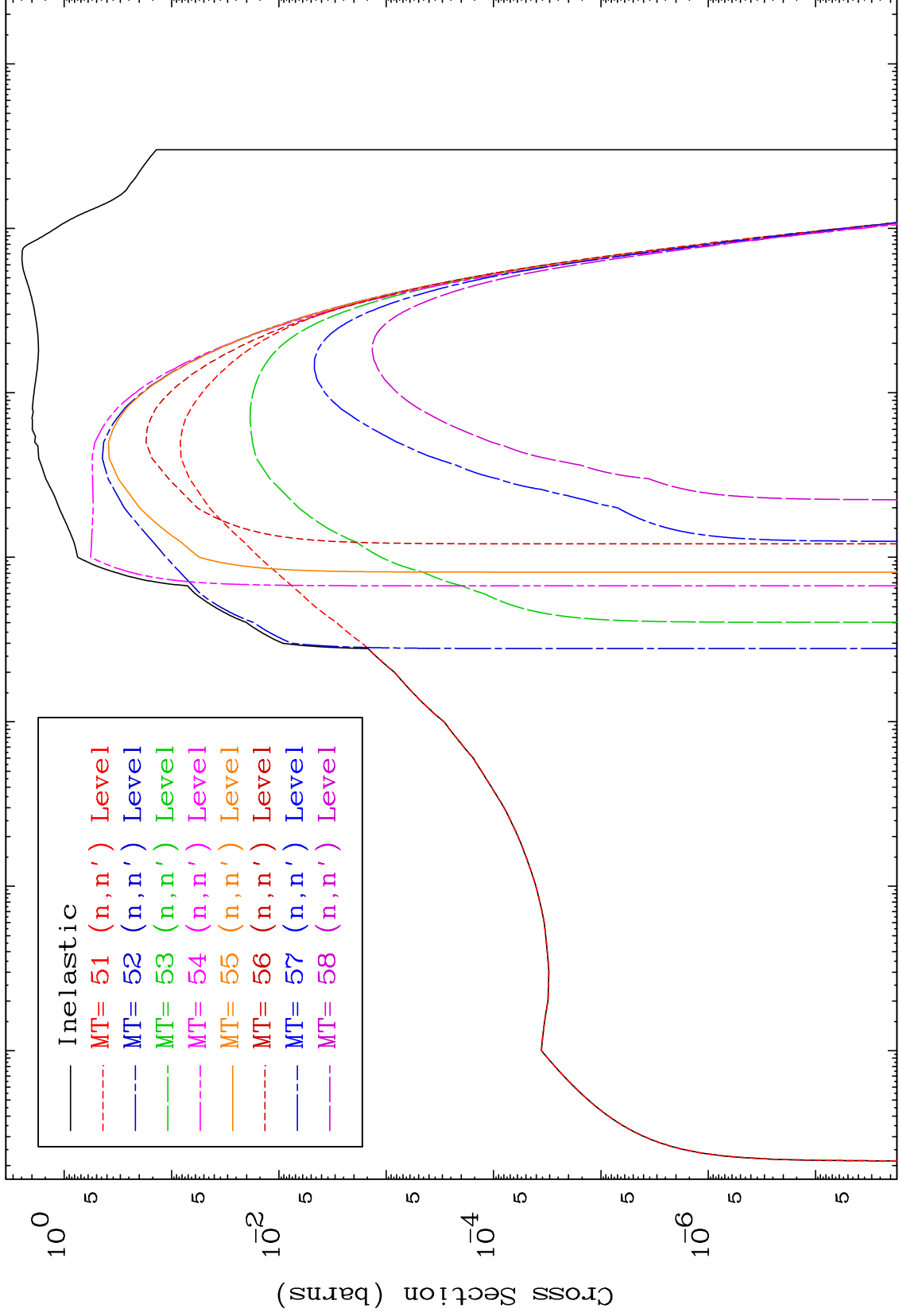


MAT 7217

(n,n') Levels

72-Hf-171m

293 Kelvin Cross Sections



10

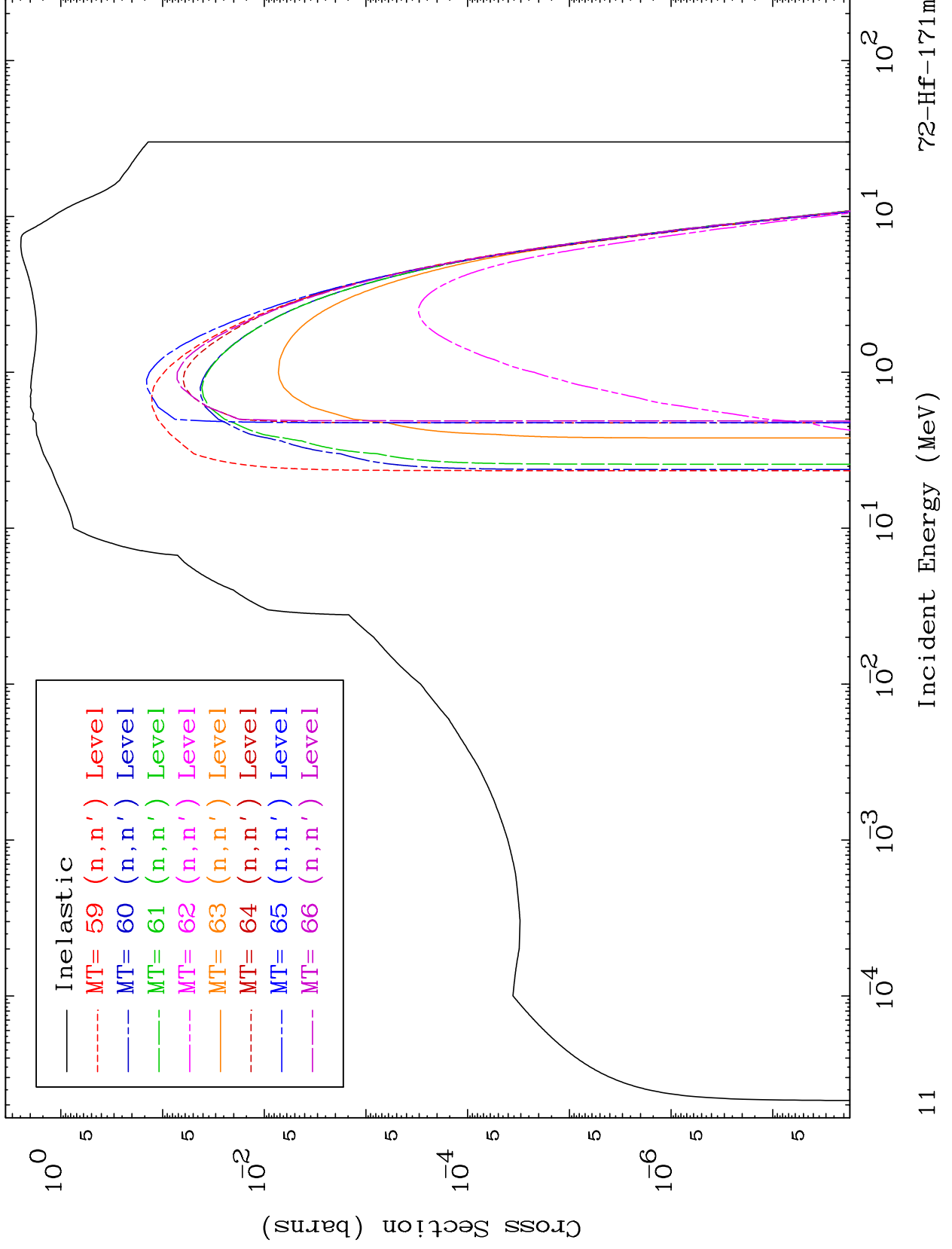
Incident Energy (MeV)

72-Hf-171m

MAT 7217

293 (n,n') Levels

72-Hf-171m

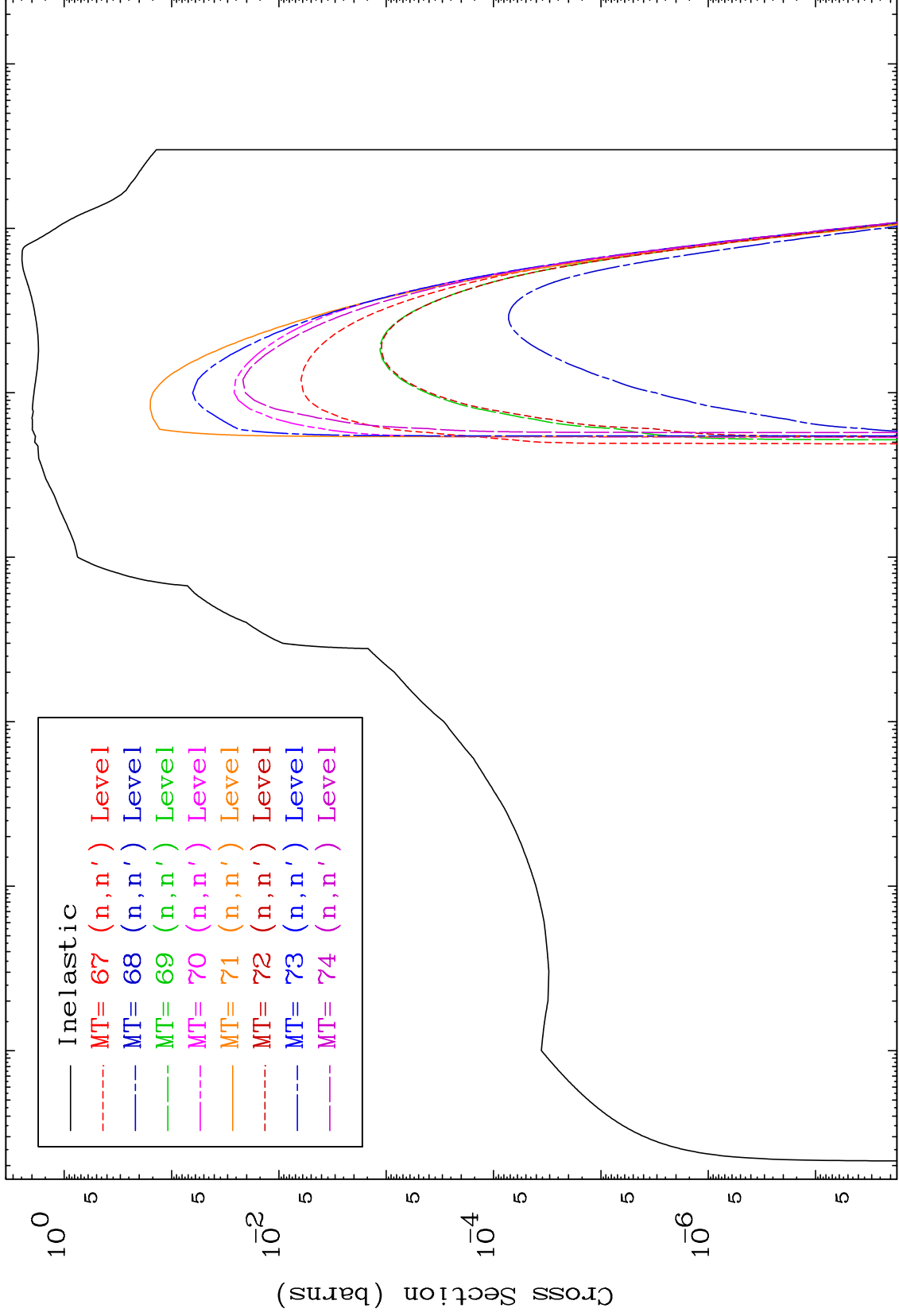


MAT 7217

(n,n') Levels

72-Hf-171m

293 Kelvin Cross Sections



Incident Energy (MeV)

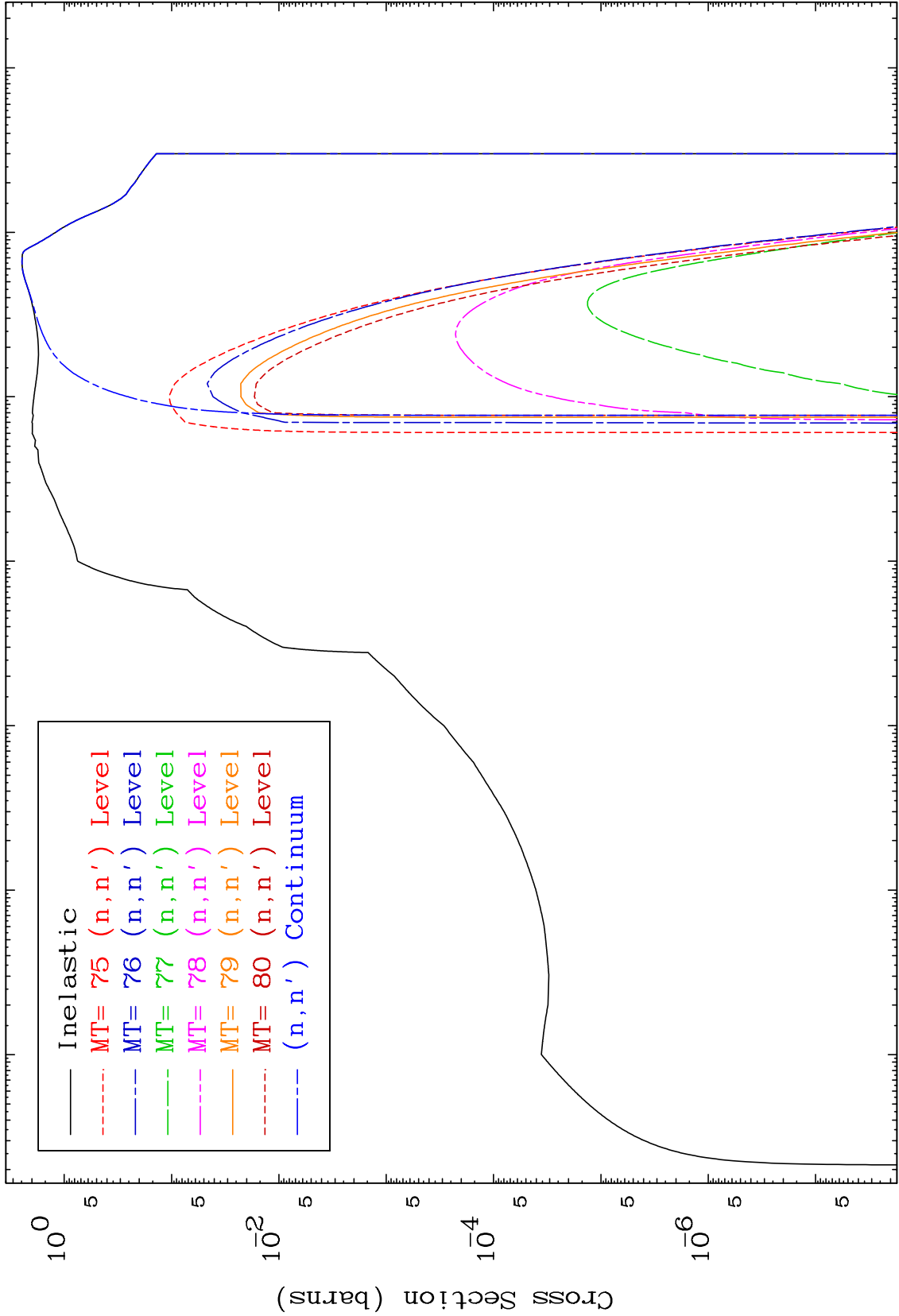
72-Hf-171m

MAT 7217

(n,n') Levels

72-Hf-171m

293 Kelvin Cross Sections



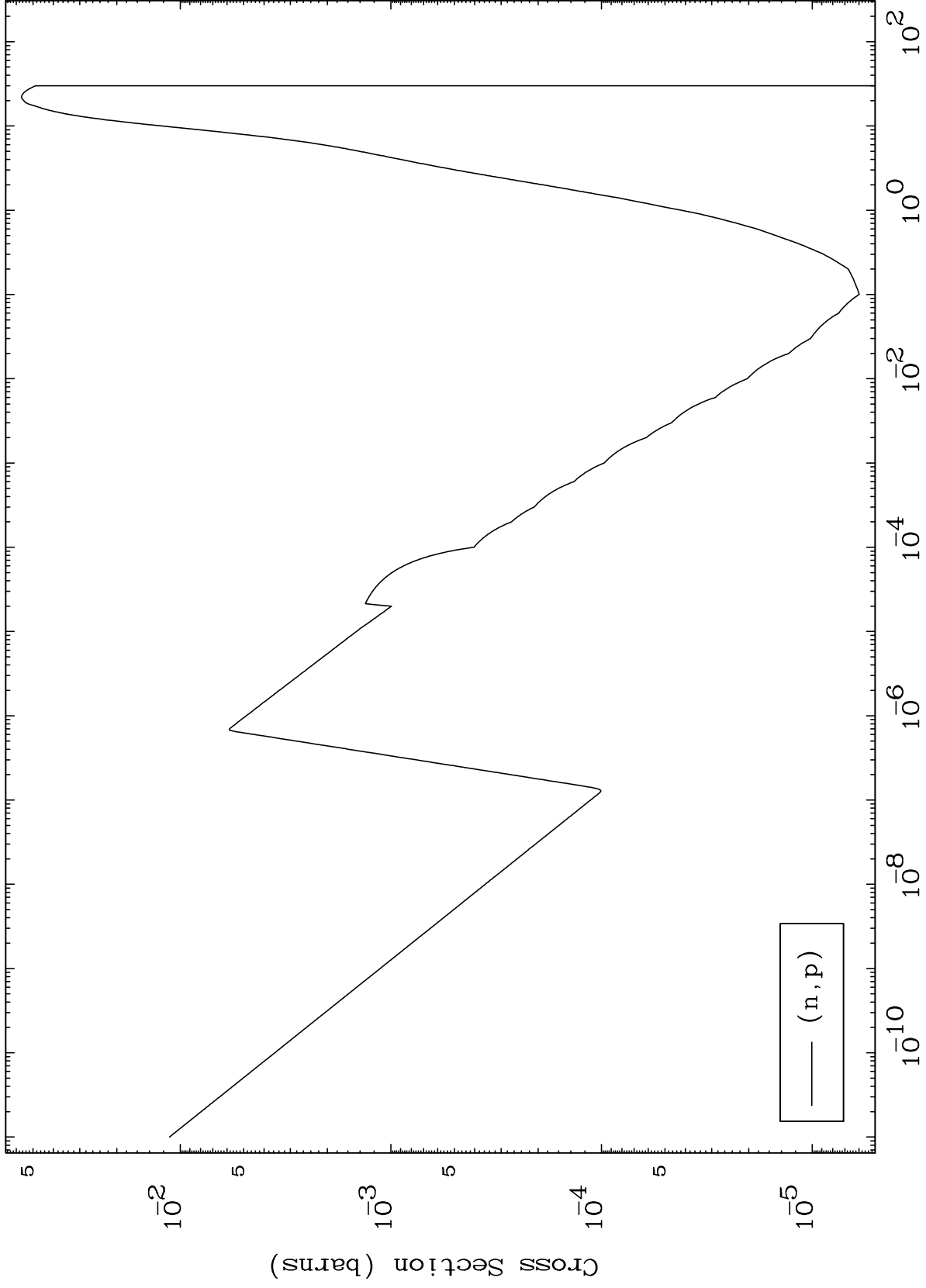
72-Hf-171m

Incident Energy (MeV)

MAT 7217

(n,p) Levels
293 Kelvin Cross Sections

72-Hf-171m



(n,p)

72-Hf-171m

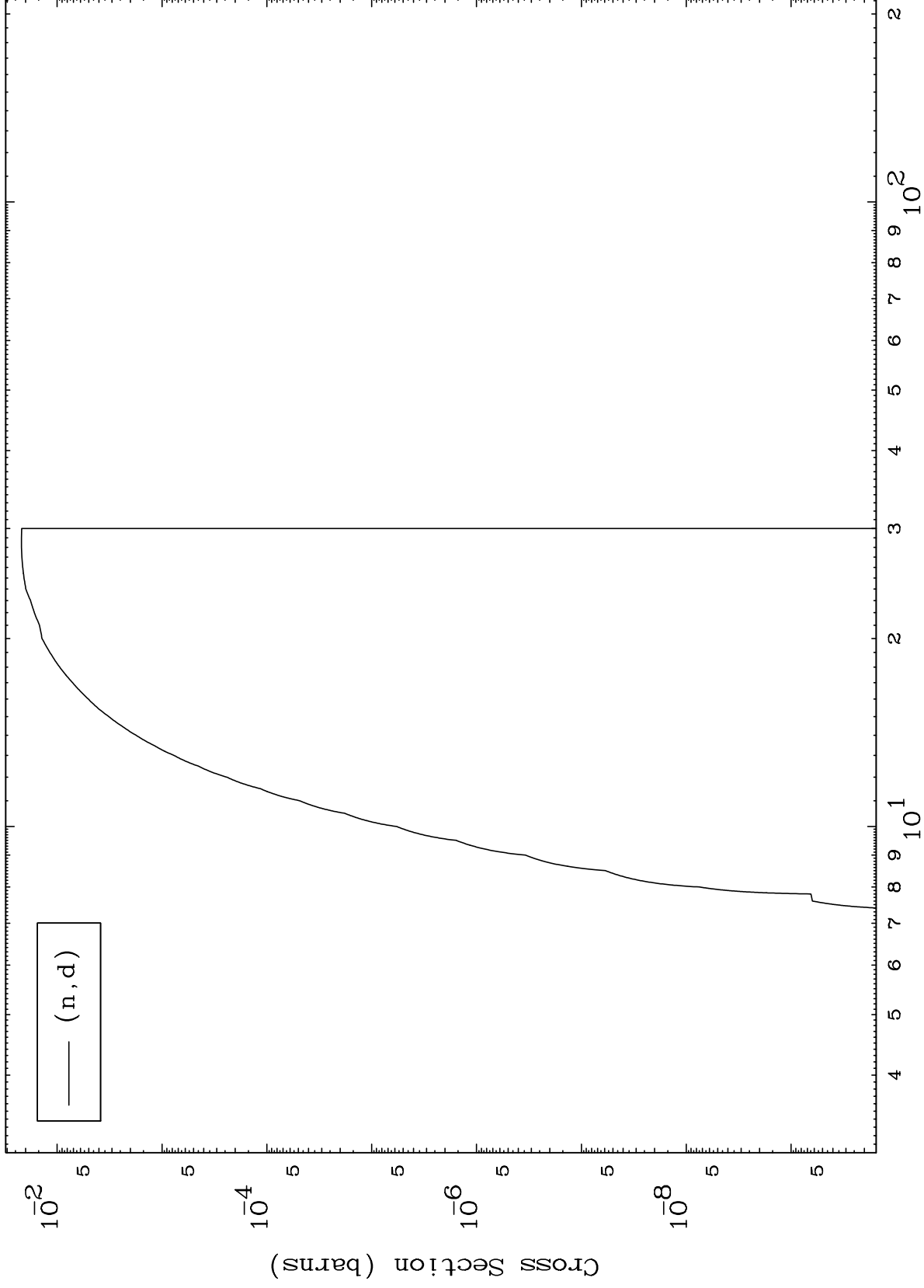
Incident Energy (MeV)

14

MAT 7217

(n,d) Levels
293 Kelvin Cross Sections

72-Hf-171m



15

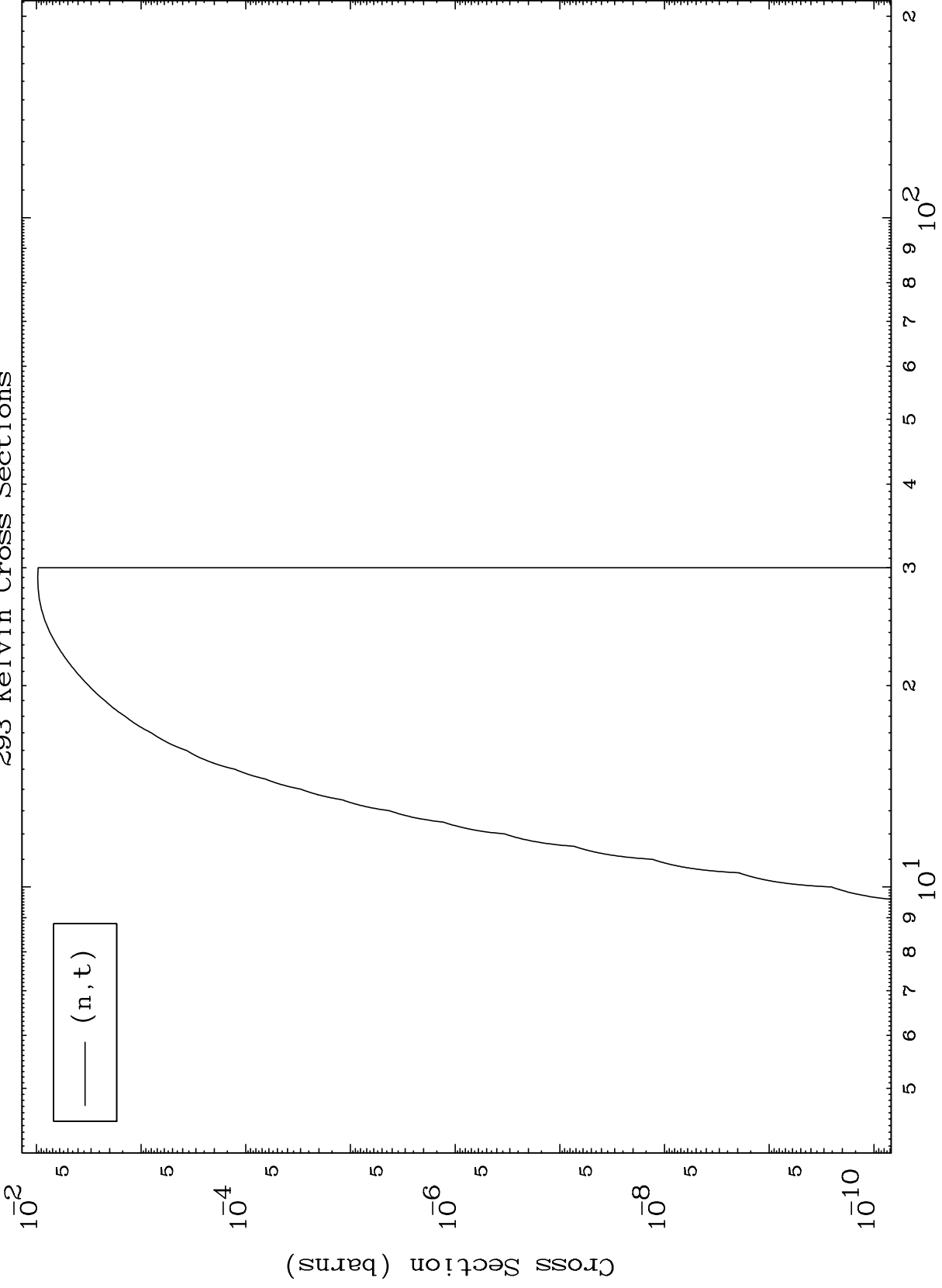
Incident Energy (MeV)

72-Hf-171m

MAT 7217

(n,t) Levels
293 Kelvin Cross Sections

72-Hf-171m



16

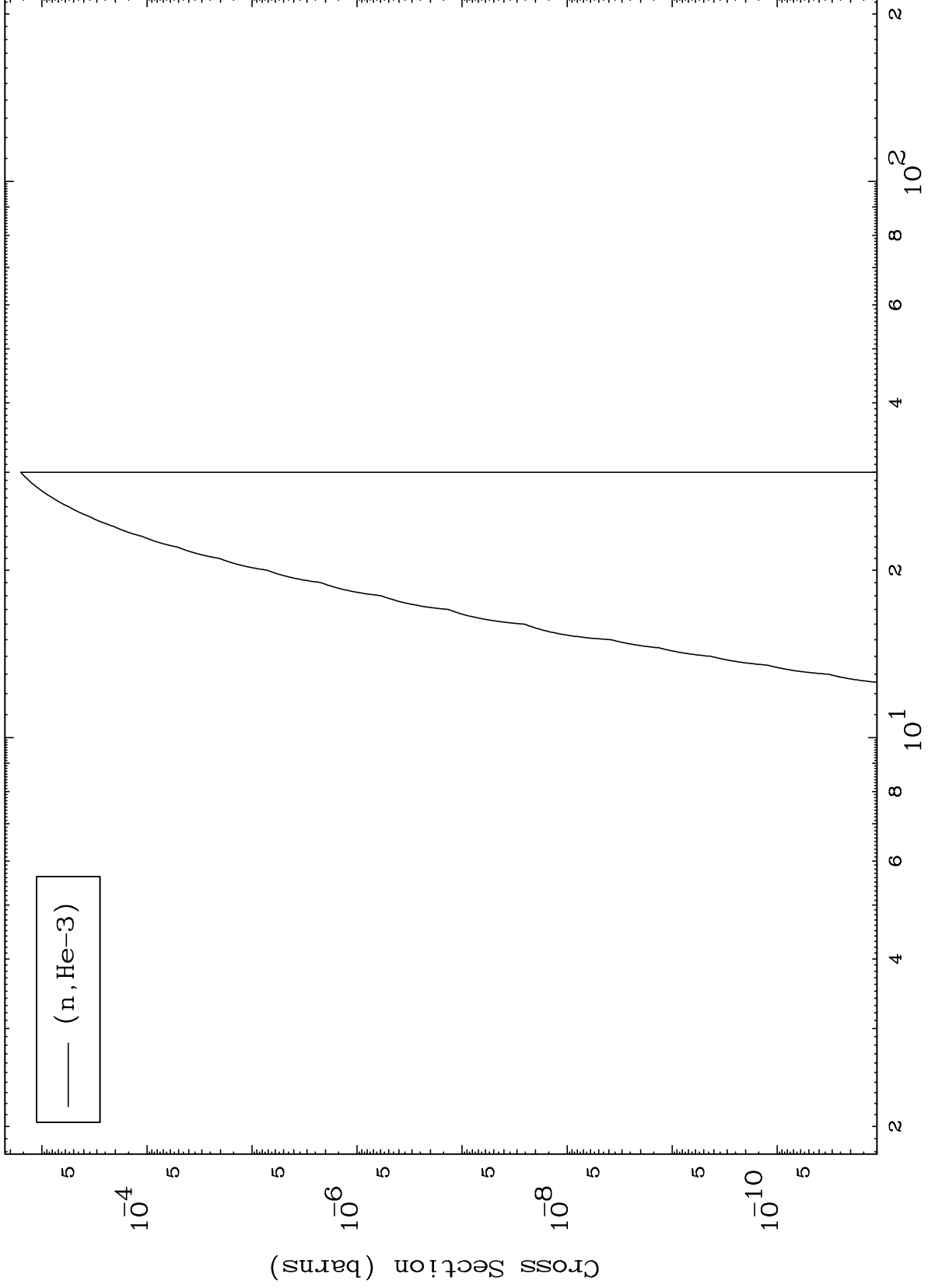
Incident Energy (MeV)

72-Hf-171m

MAT 7217

(n,He3) Levels
293 Kelvin Cross Sections

72-Hf-171m



17

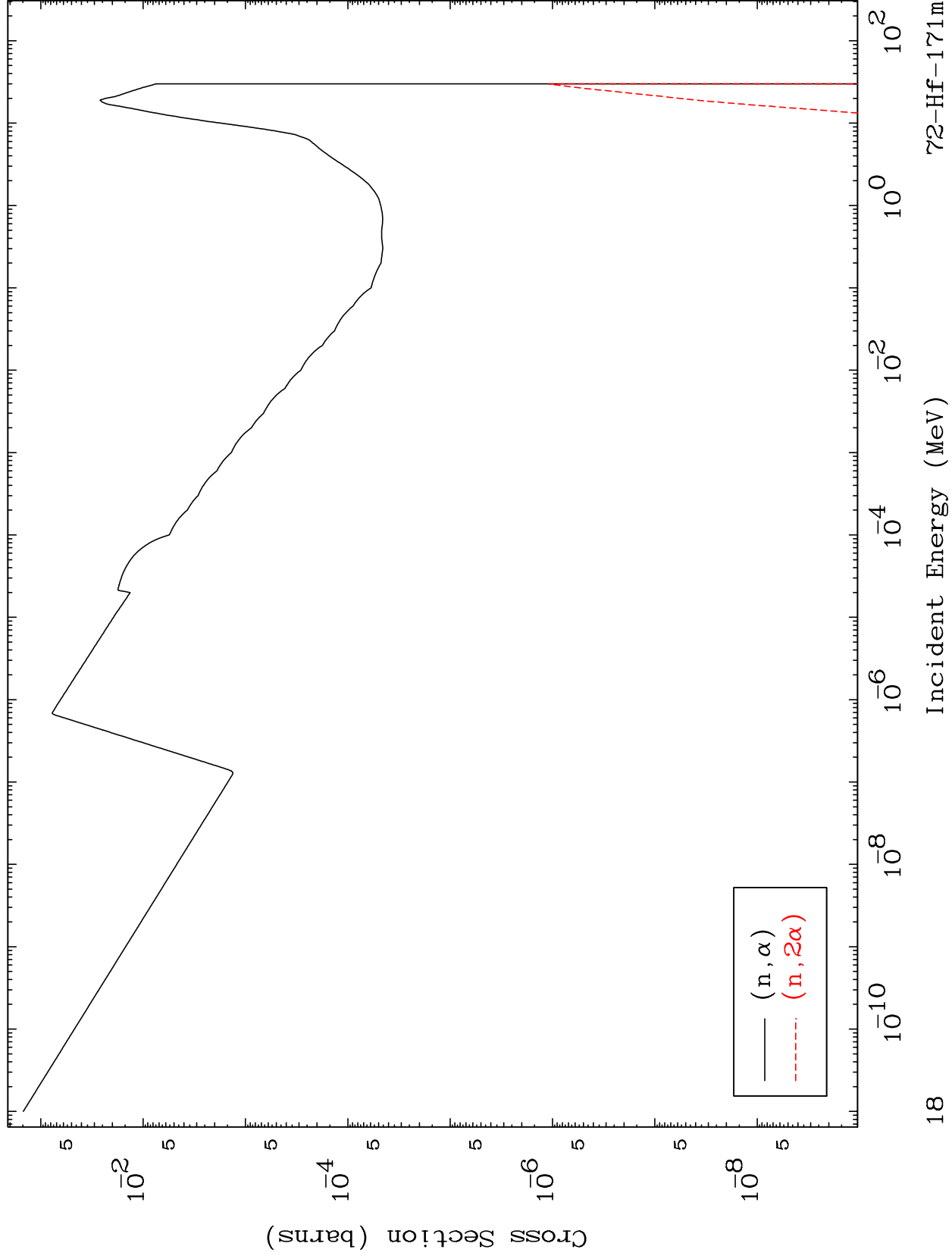
Incident Energy (MeV)

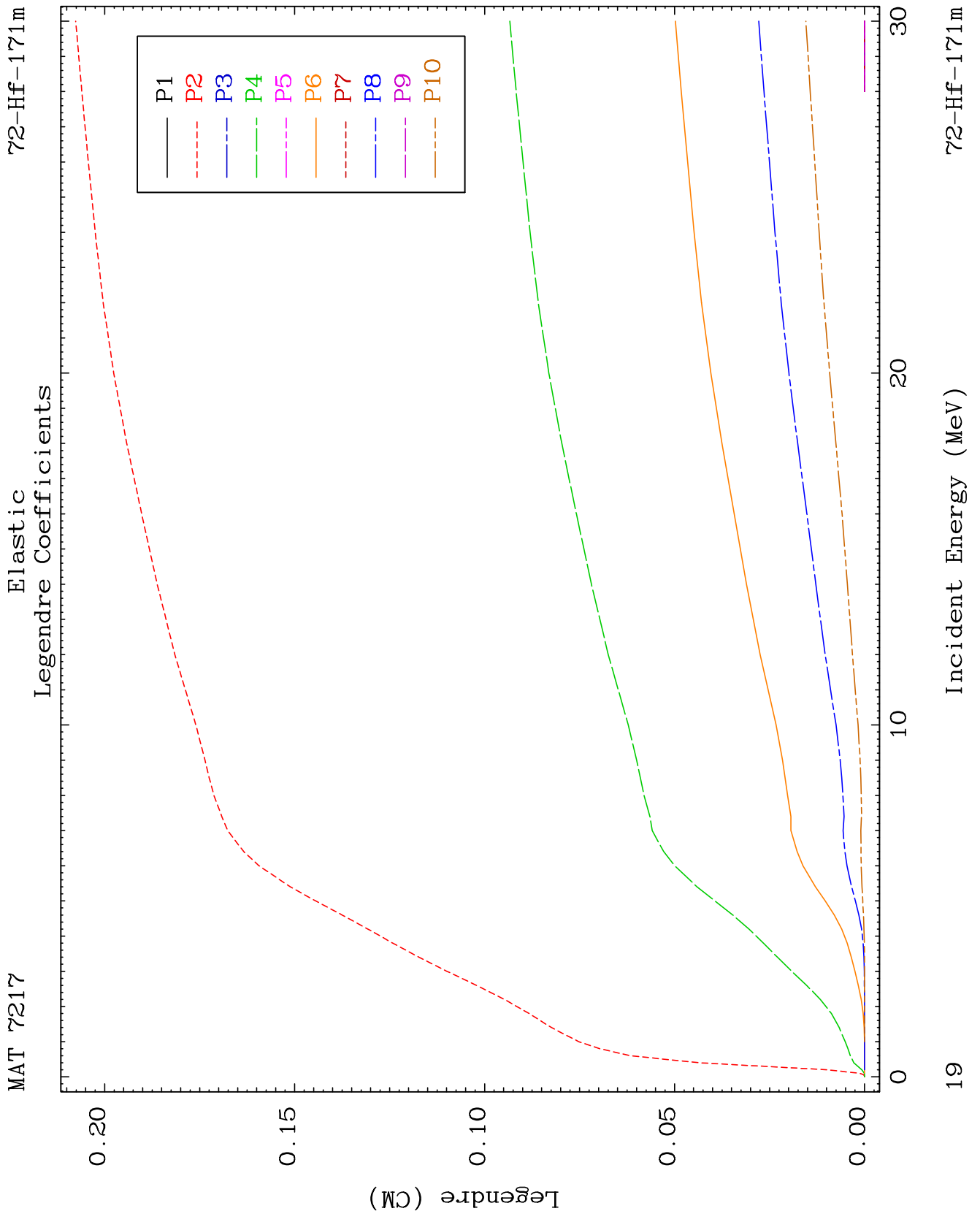
72-Hf-171m

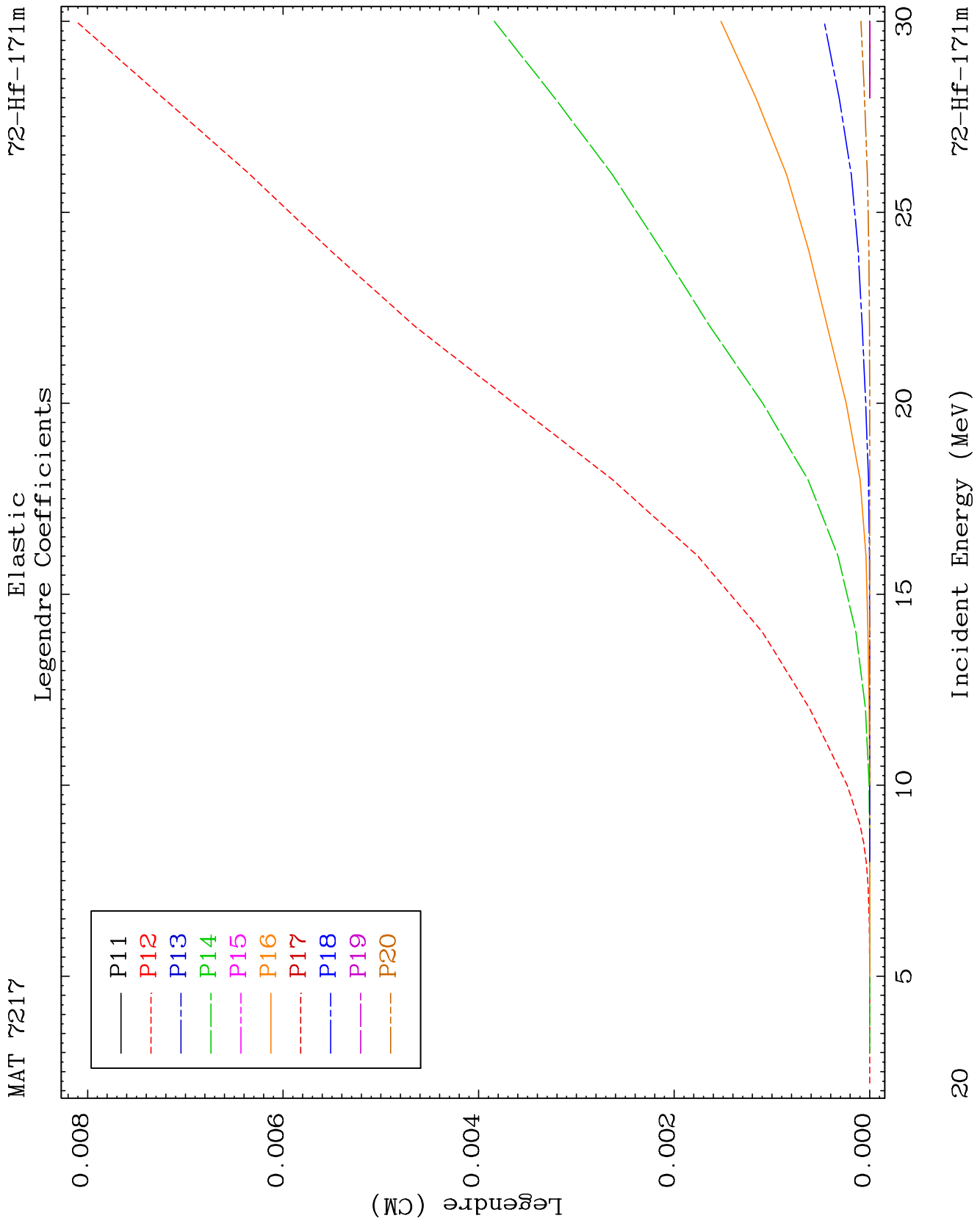
MAT 7217

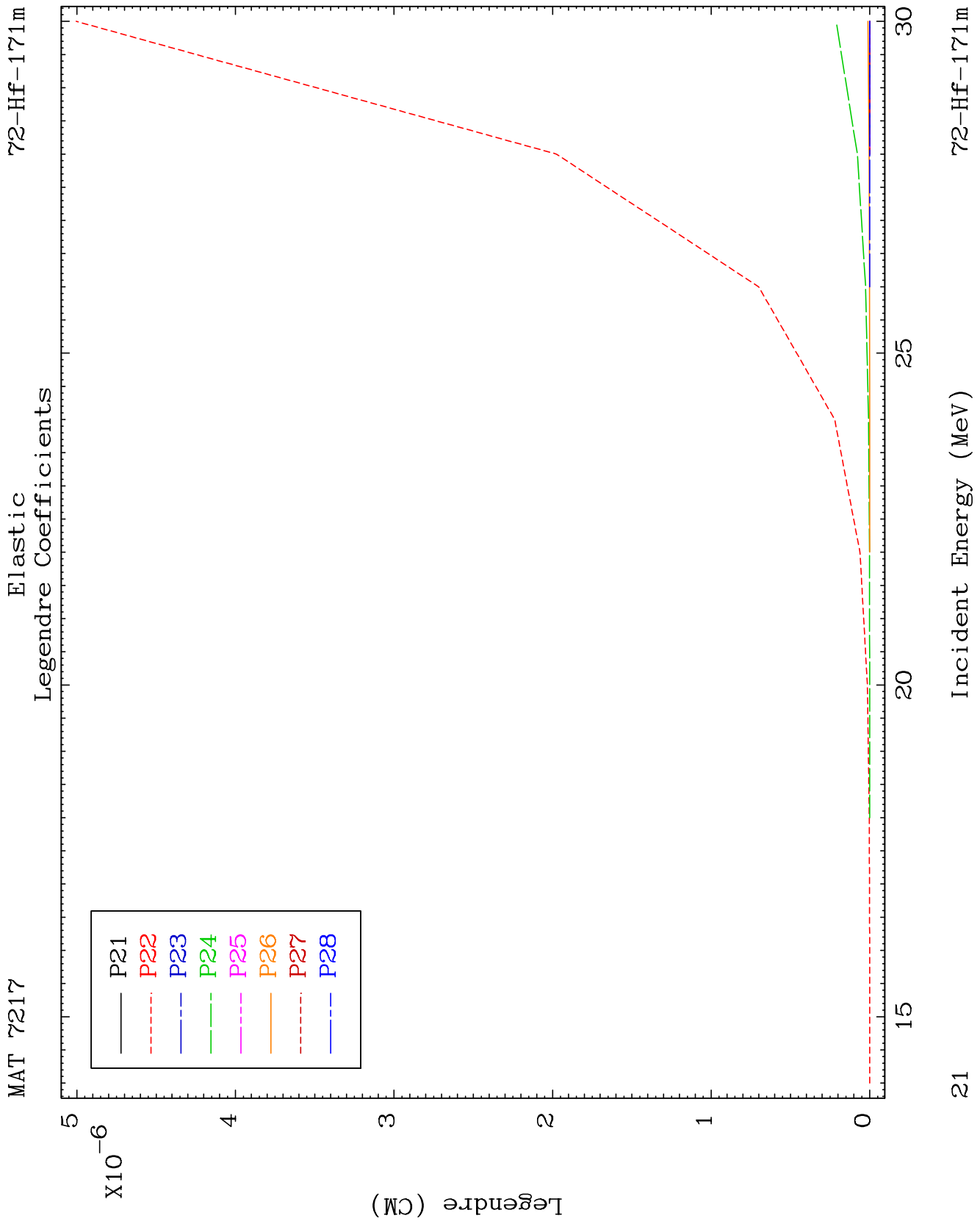
(n, α) Levels
293 Kelvin Cross Sections

⁷²Hf-171m





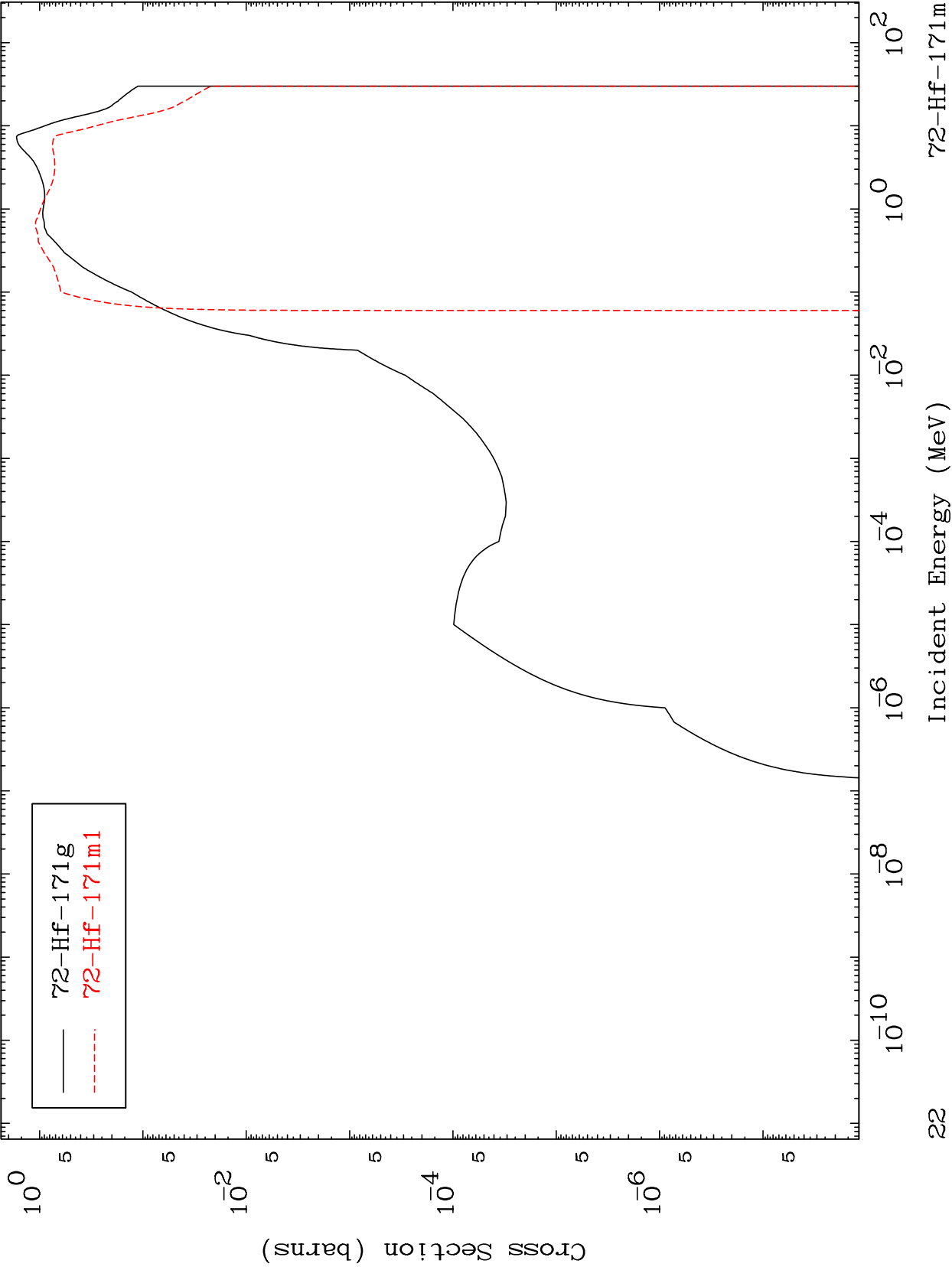




MAT 7217

Inelastic
Radionuclide Production Cross Section

⁷²Hf-171m

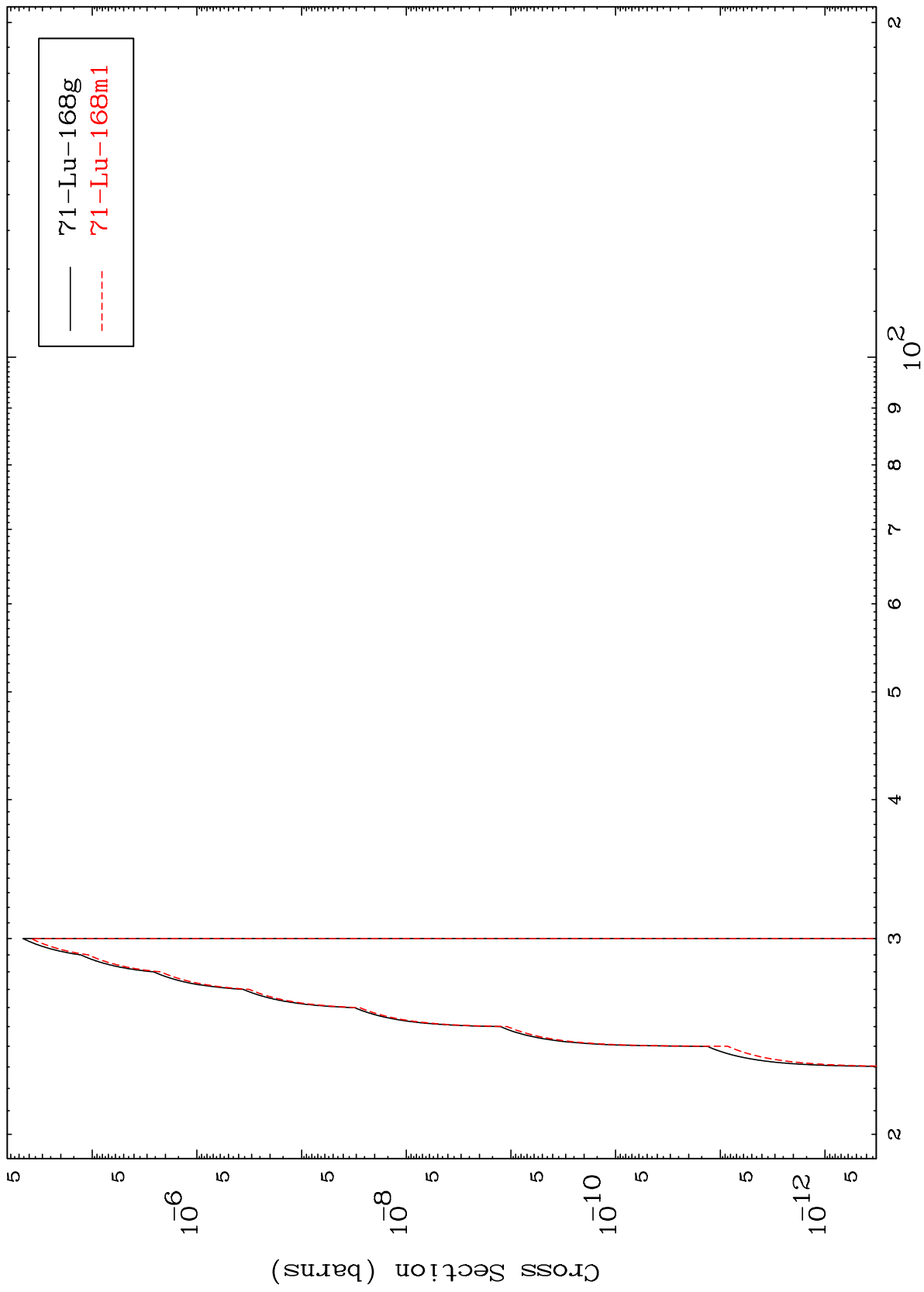


MAT 7217

(n,2n) d

72-Hf-171m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

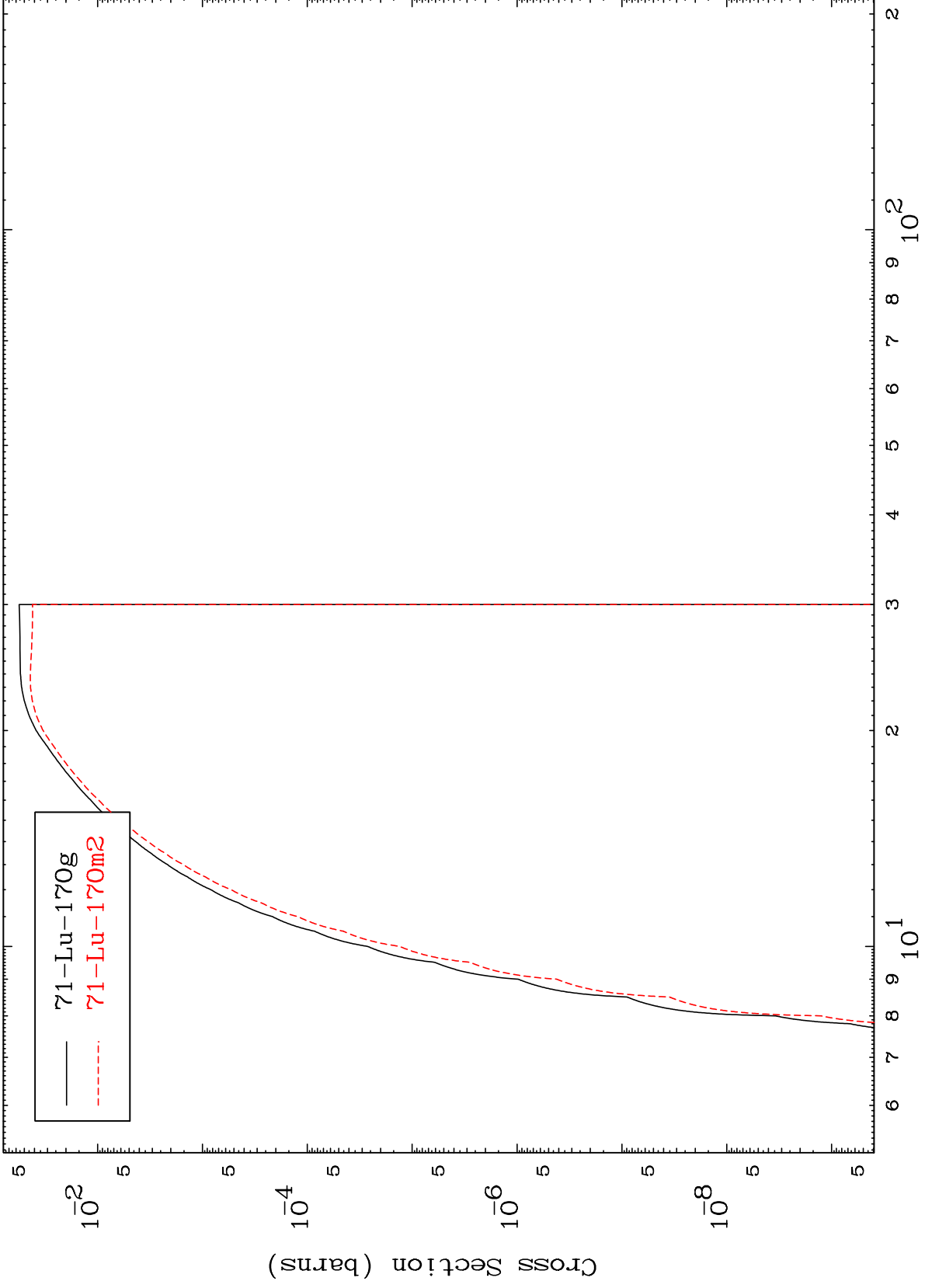
72-Hf-171m

MAT 7217

(n,n') p

72-Hf-171m

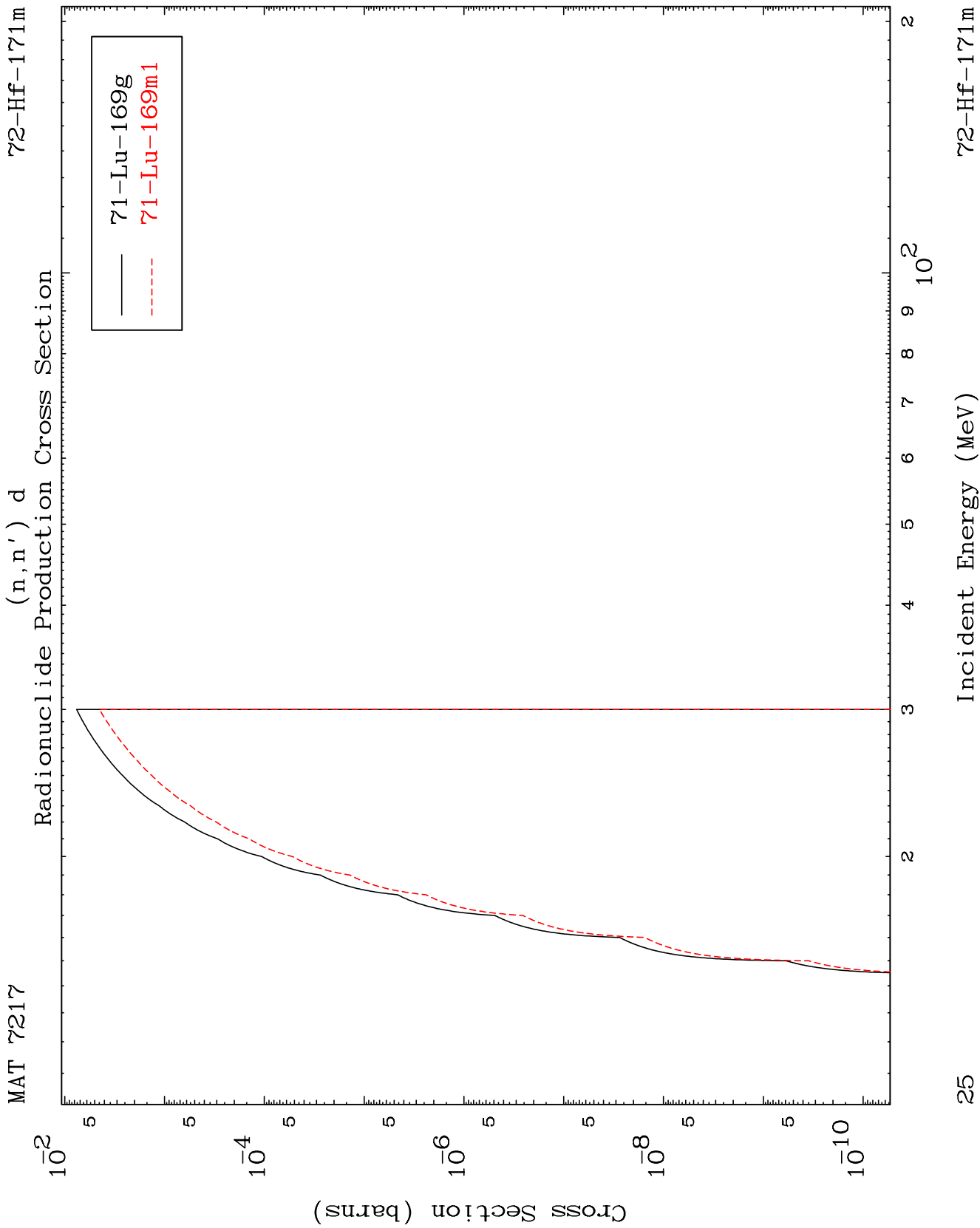
Radionuclide Production Cross Section



24

Incident Energy (MeV)

72-Hf-171m

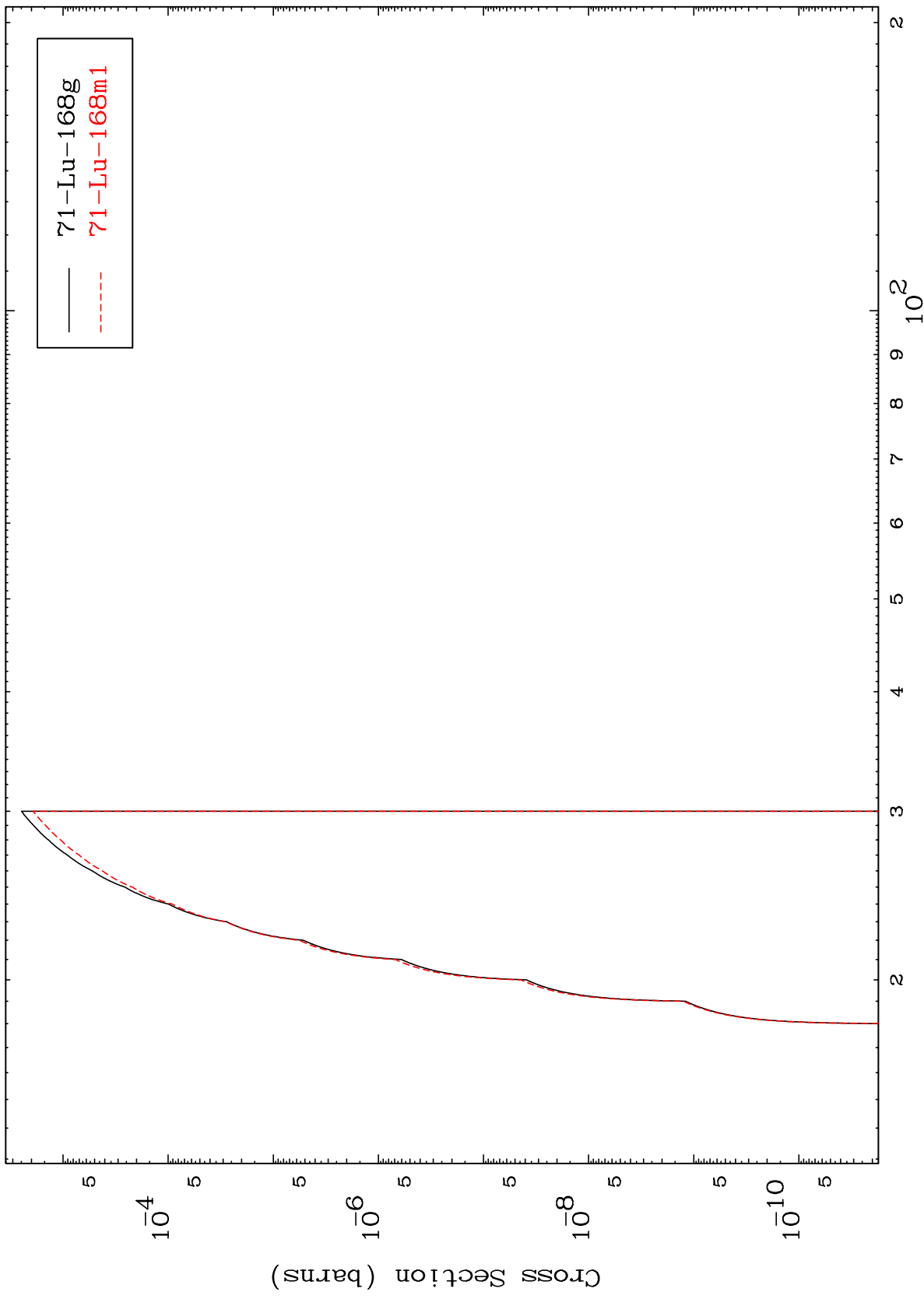


MAT 7217

(n,n') t

72-Hf-171m

Radionuclide Production Cross Section



26

Incident Energy (MeV)

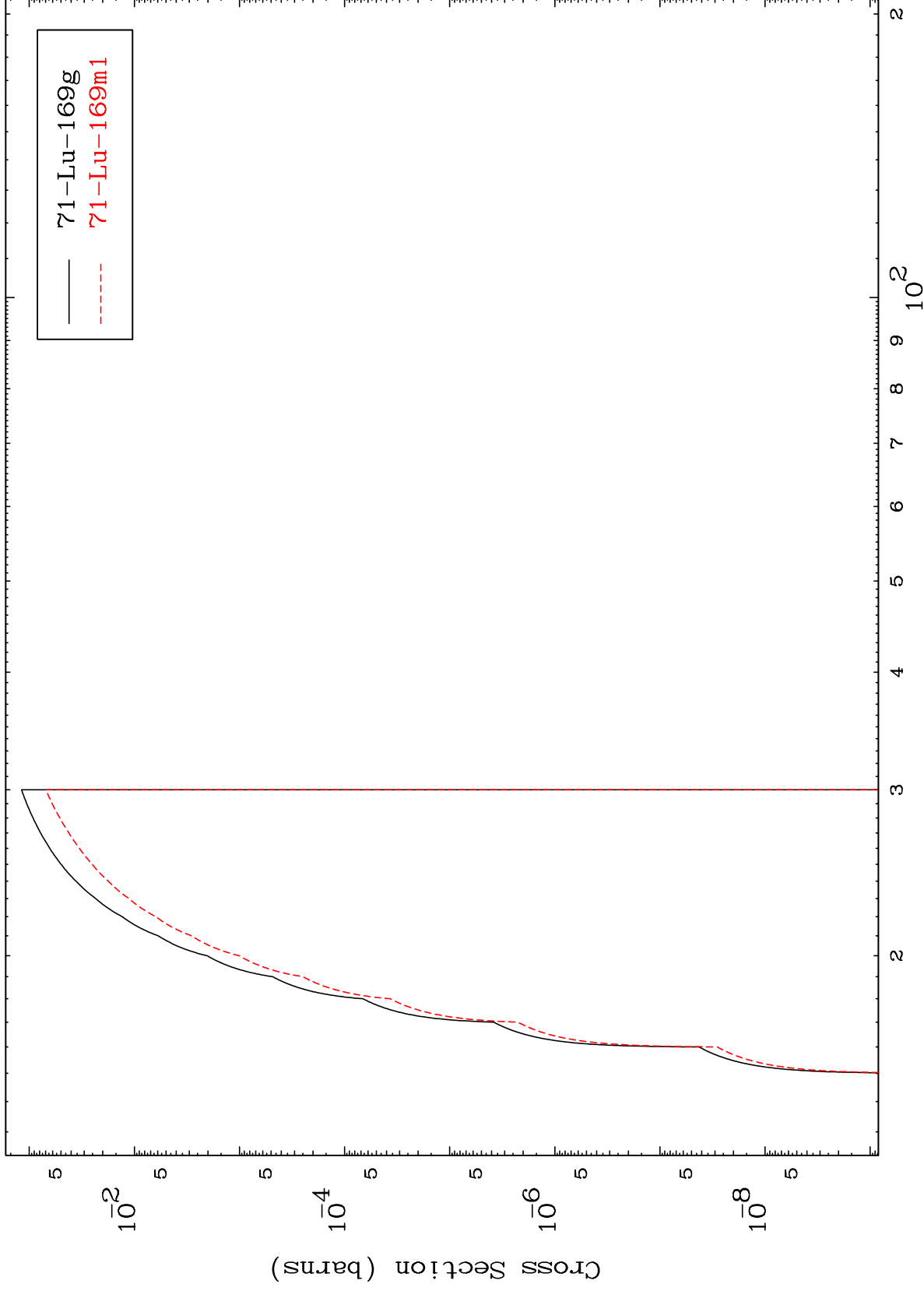
72-Hf-171m

MAT 7217

(n,2n) p

72-Hf-171m

Radionuclide Production Cross Section



27

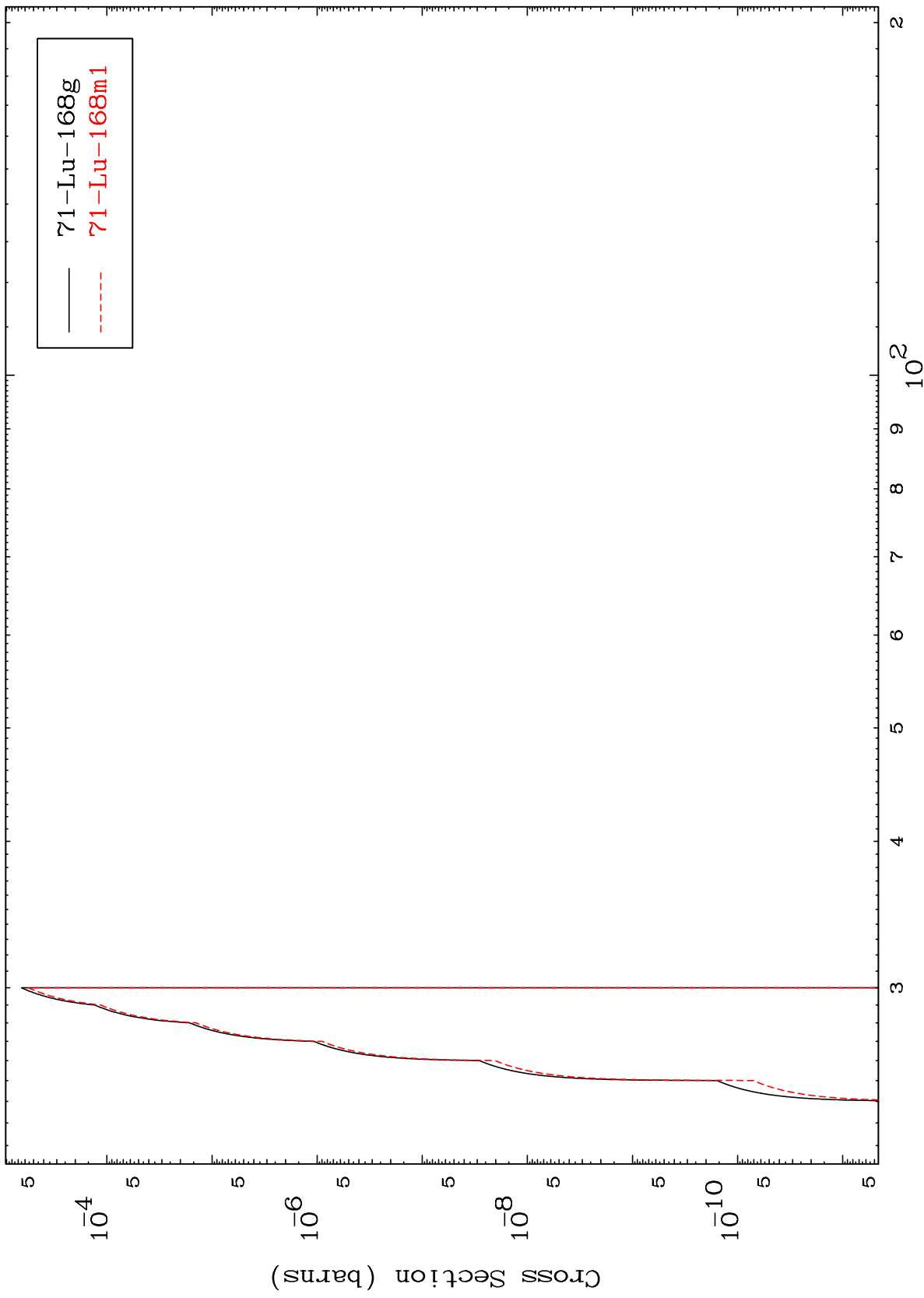
Incident Energy (MeV)

72-Hf-171m

MAT 7217

72-Hf-171m

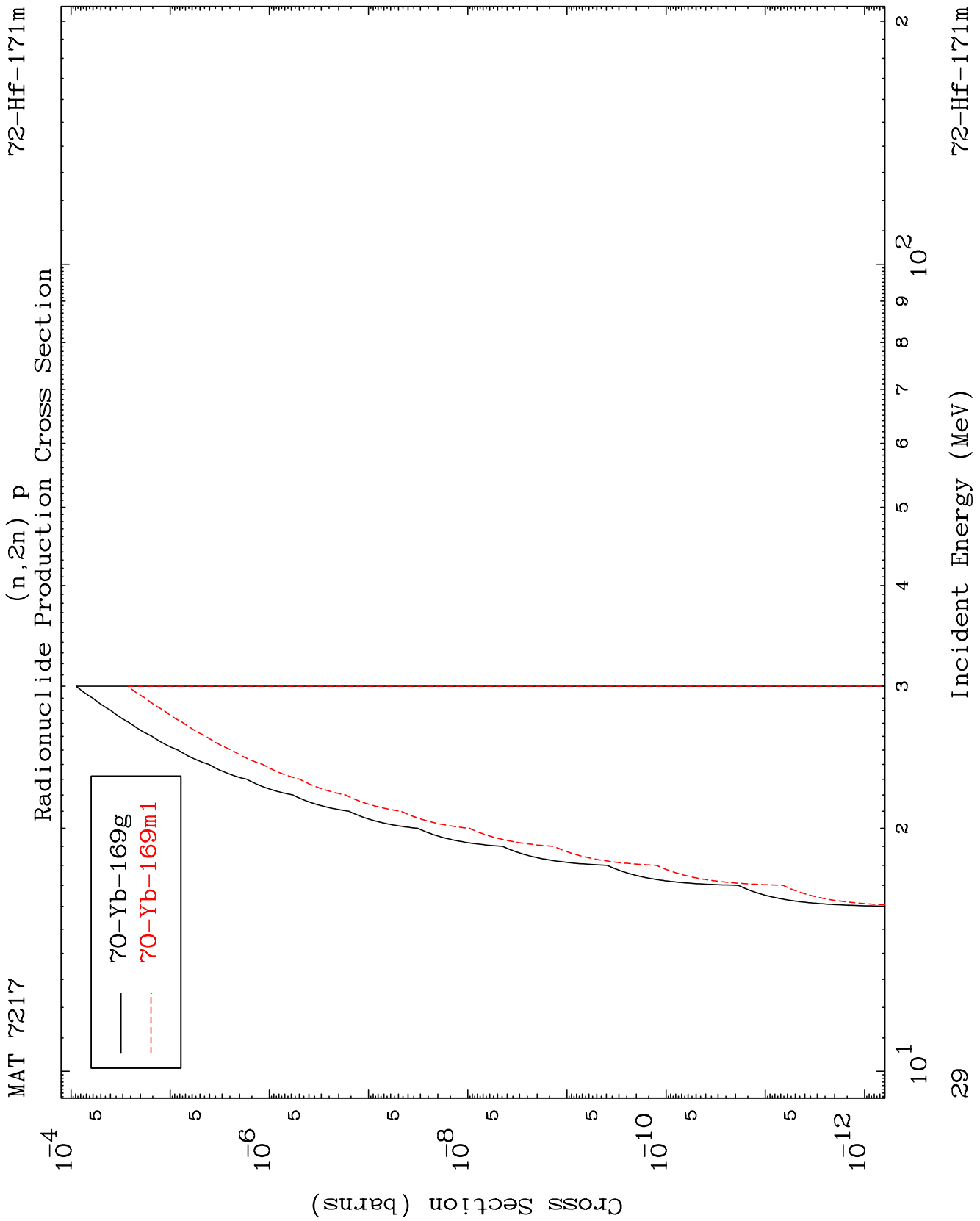
(n,3n) p
Radionuclide Production Cross Section



28

Incident Energy (MeV)

72-Hf-171m

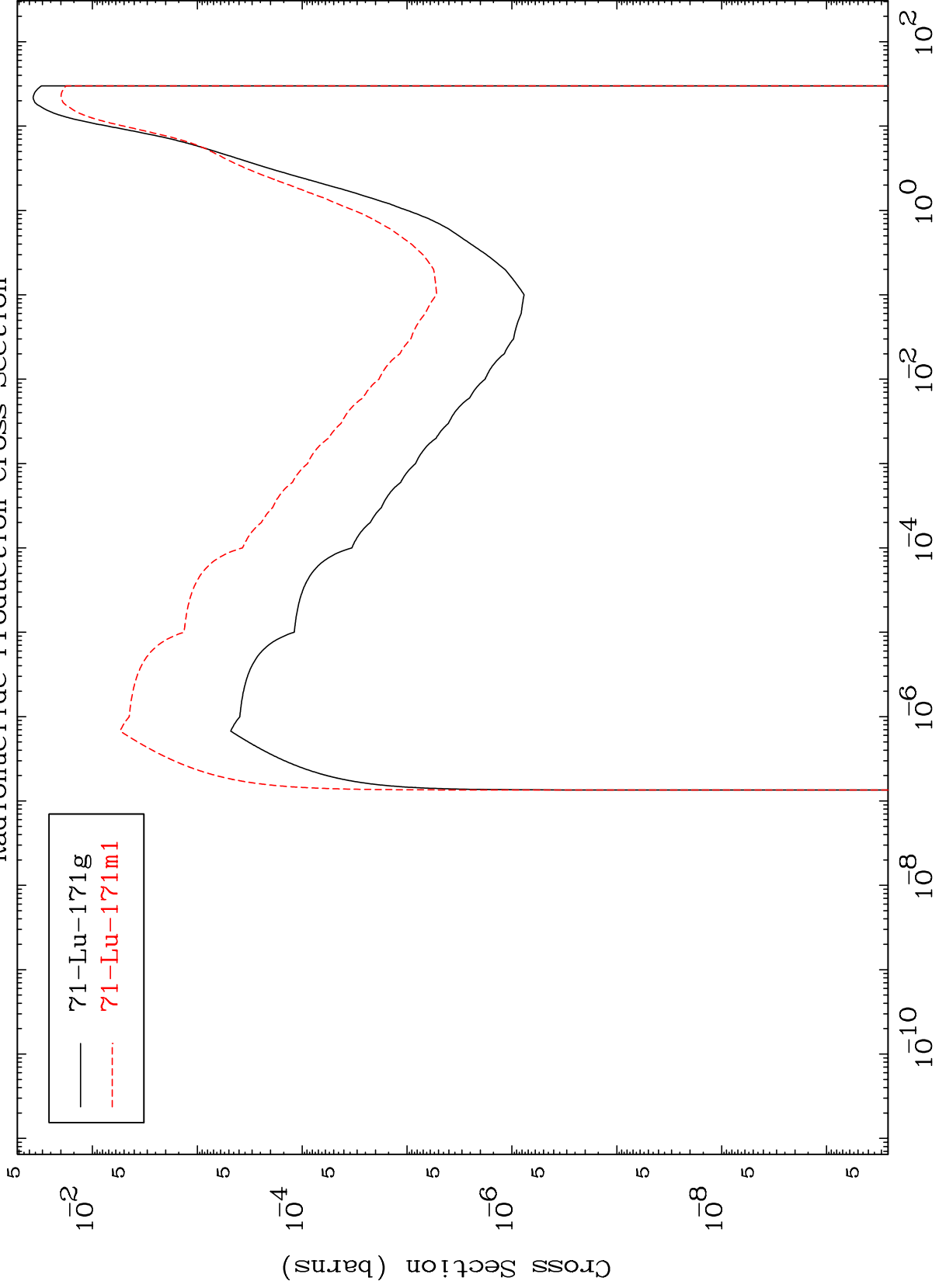


MAT 7217

(n,p)

⁷²Hf-171m

Radionuclide Production Cross Section

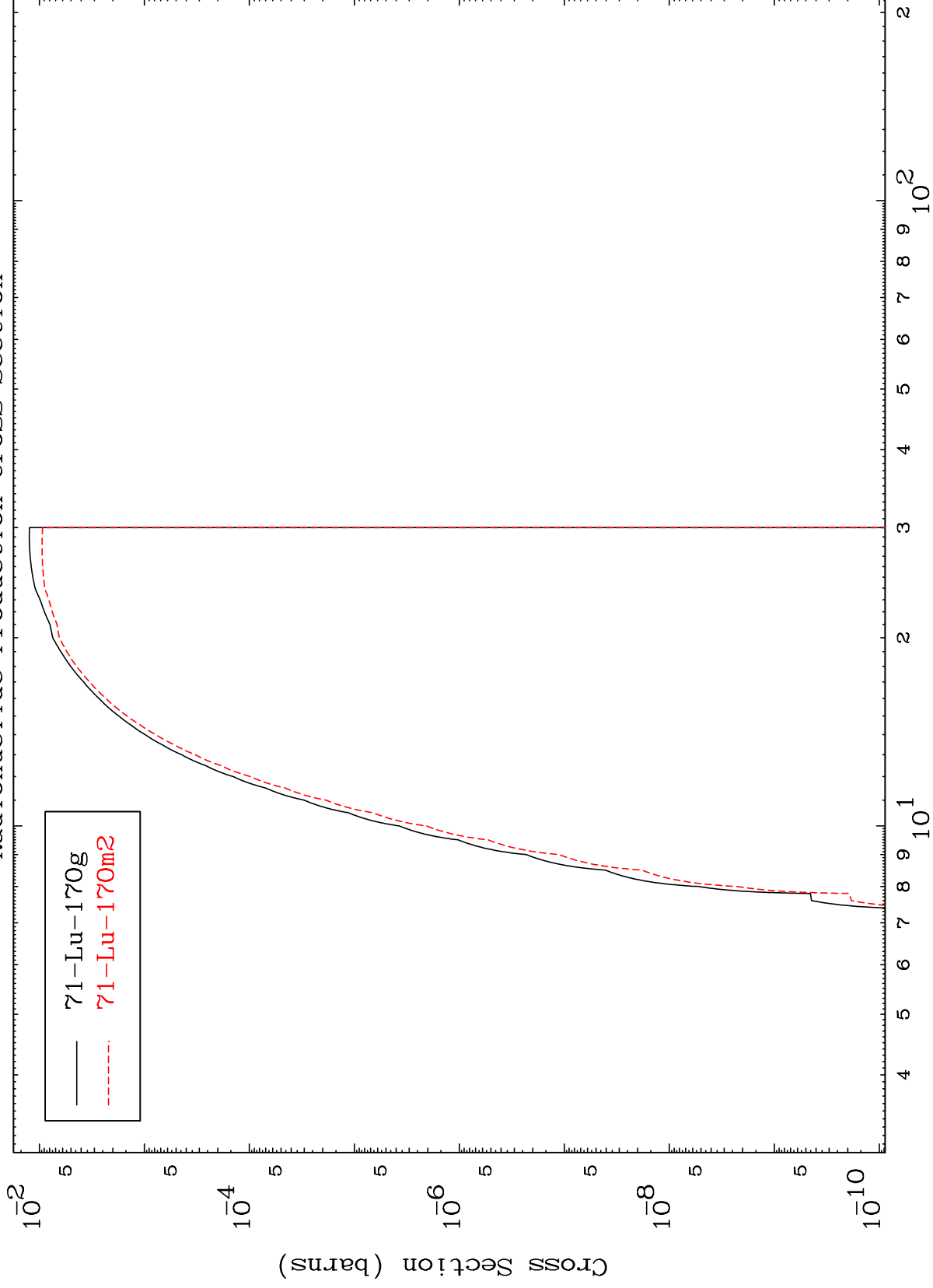


MAT 7217

(n, d)

⁷²Hf-171m

Radionuclide Production Cross Section



31

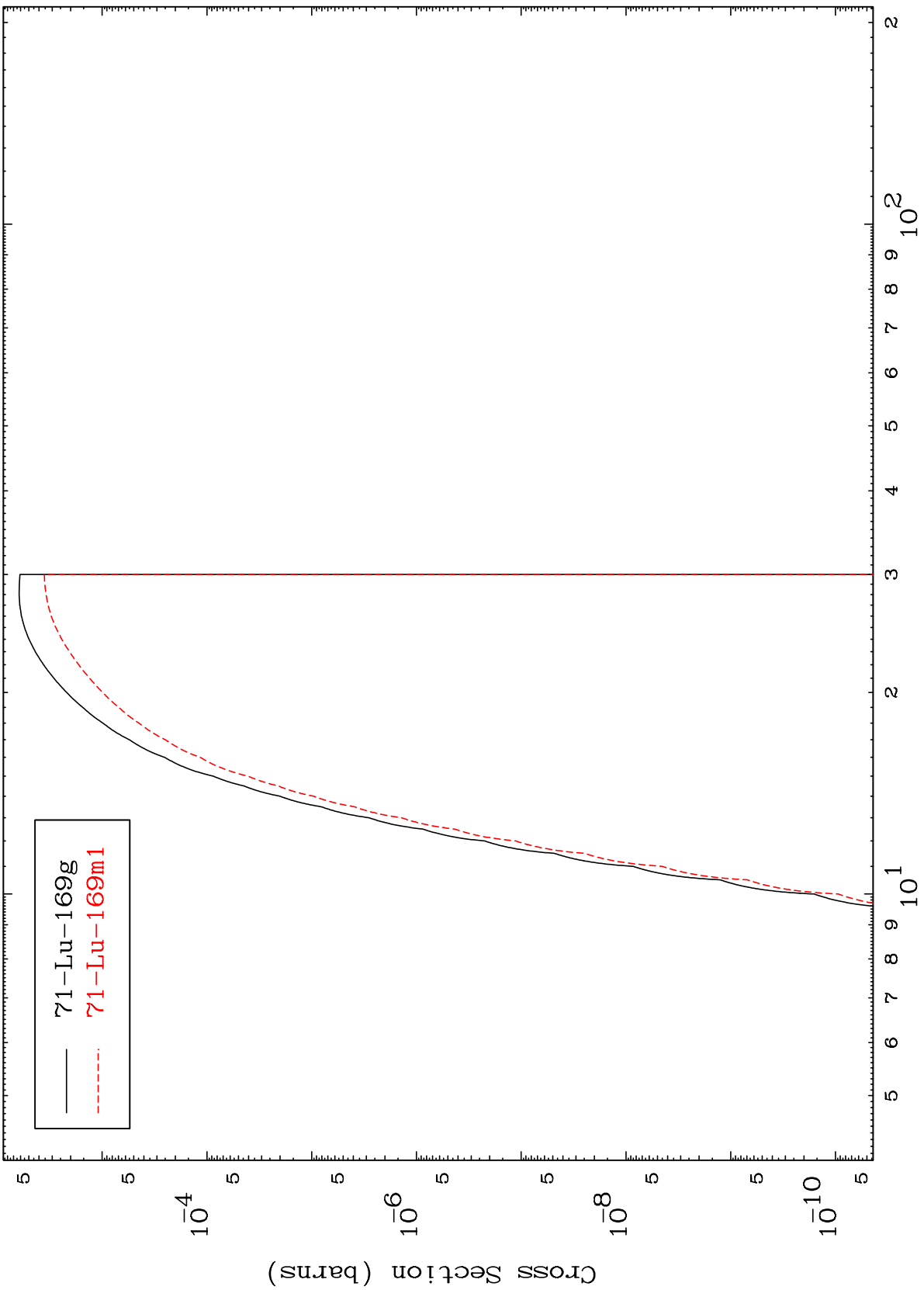
Incident Energy (MeV)

⁷²Hf-171m

MAT 7217

72-Hf-171m

Radionuclide Production Cross Section (n, t)



32

Incident Energy (MeV)

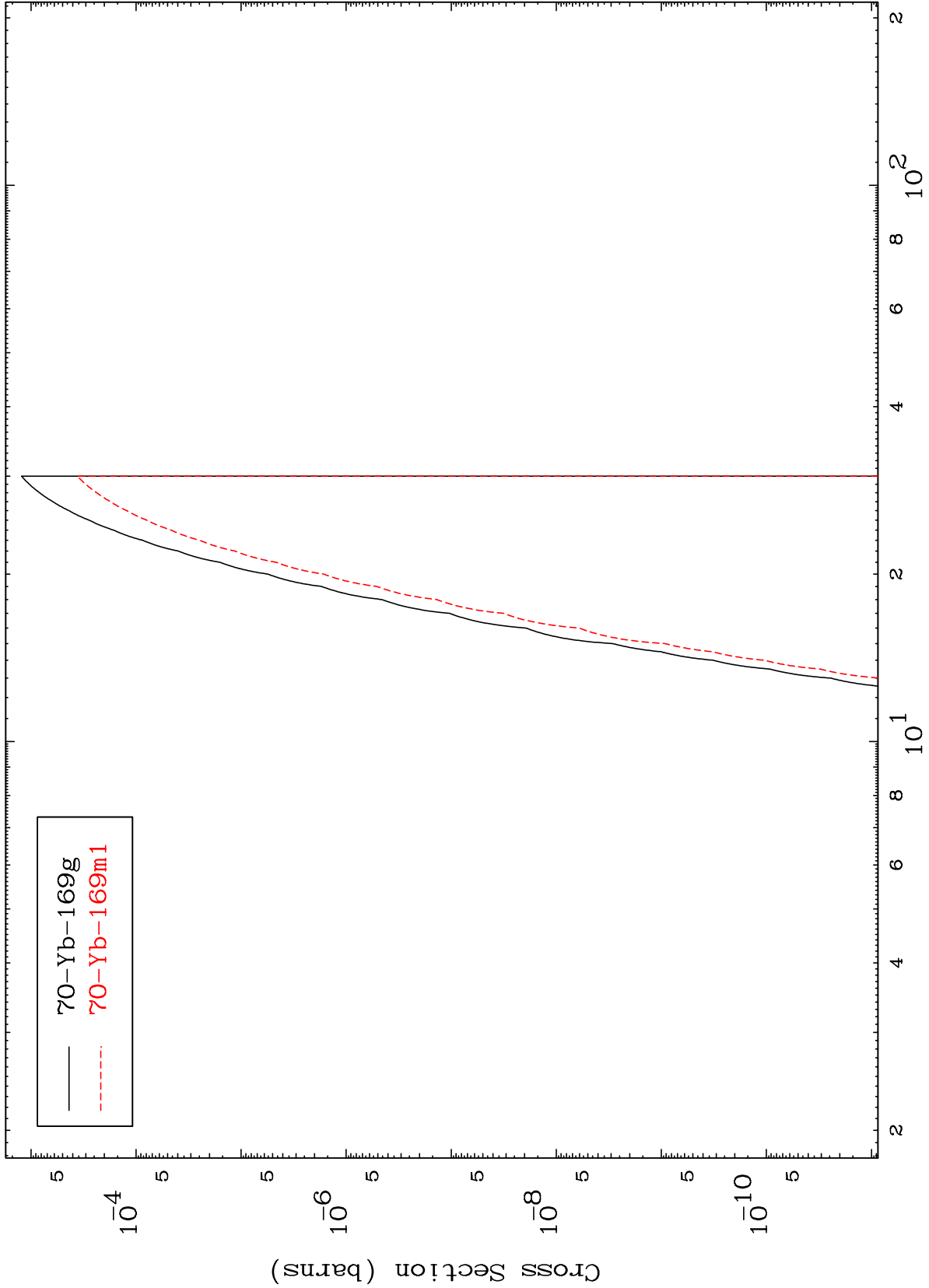
72-Hf-171m

MAT 7217

(n,He-3)

72-Hf-171m

Radionuclide Production Cross Section



33

Incident Energy (MeV)

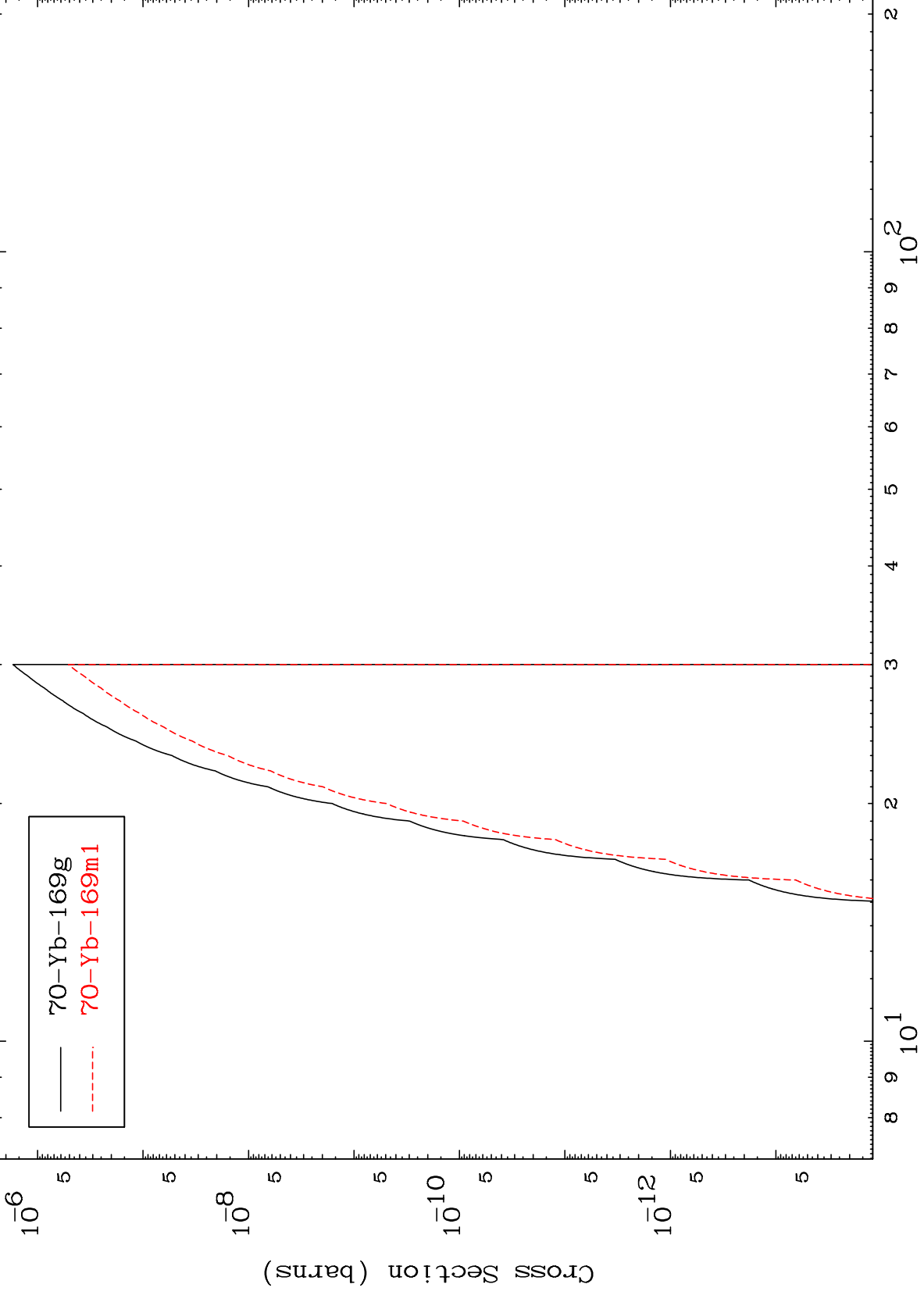
72-Hf-171m

MAT 7217

(n,p) d

⁷²Hf-171m

Radionuclide Production Cross Section



34

Incident Energy (MeV)

⁷²Hf-171m