

Program EVALPLOT
(Version 2021-1)

by

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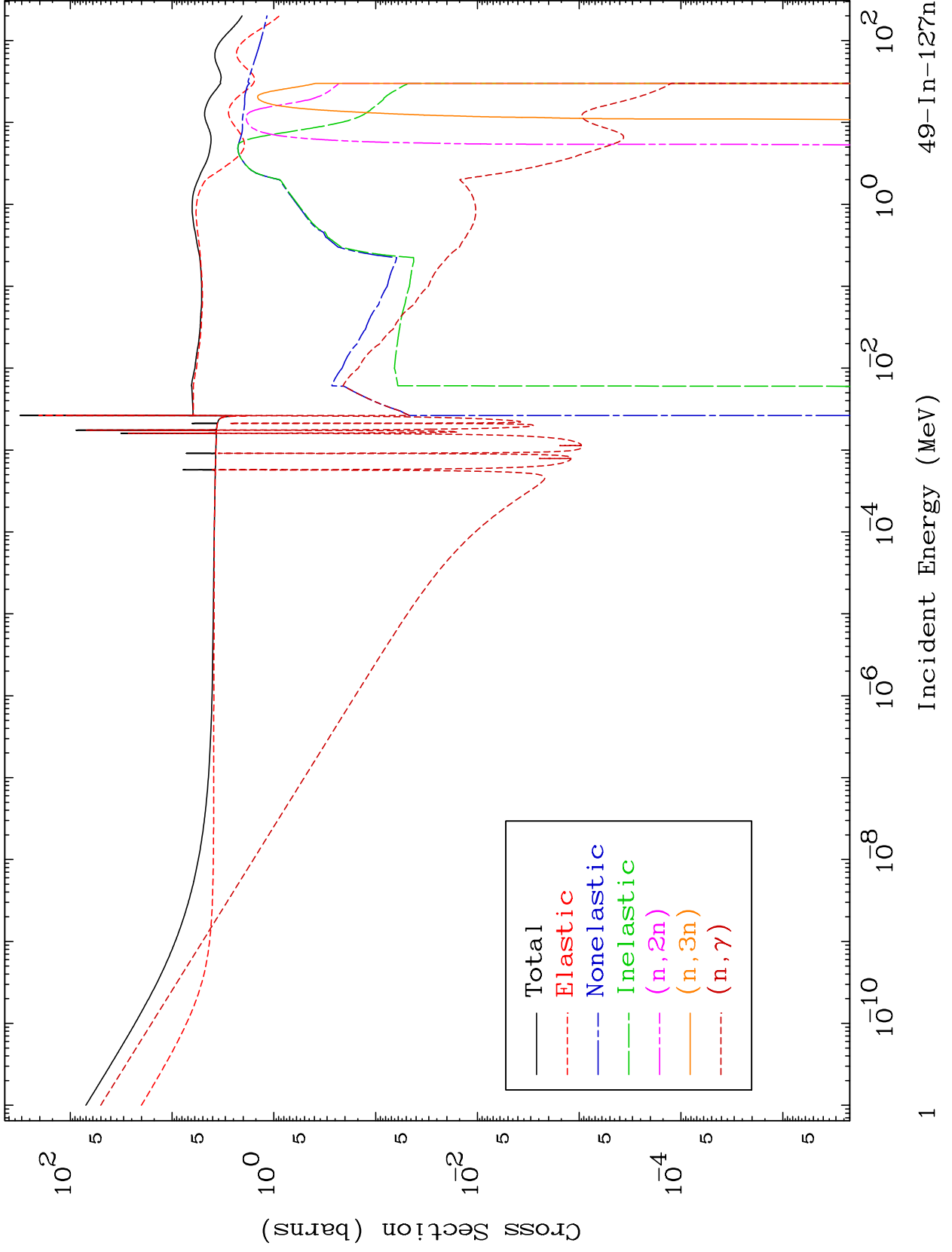
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 4969

Neutron Major
293 Kelvin Cross Sections

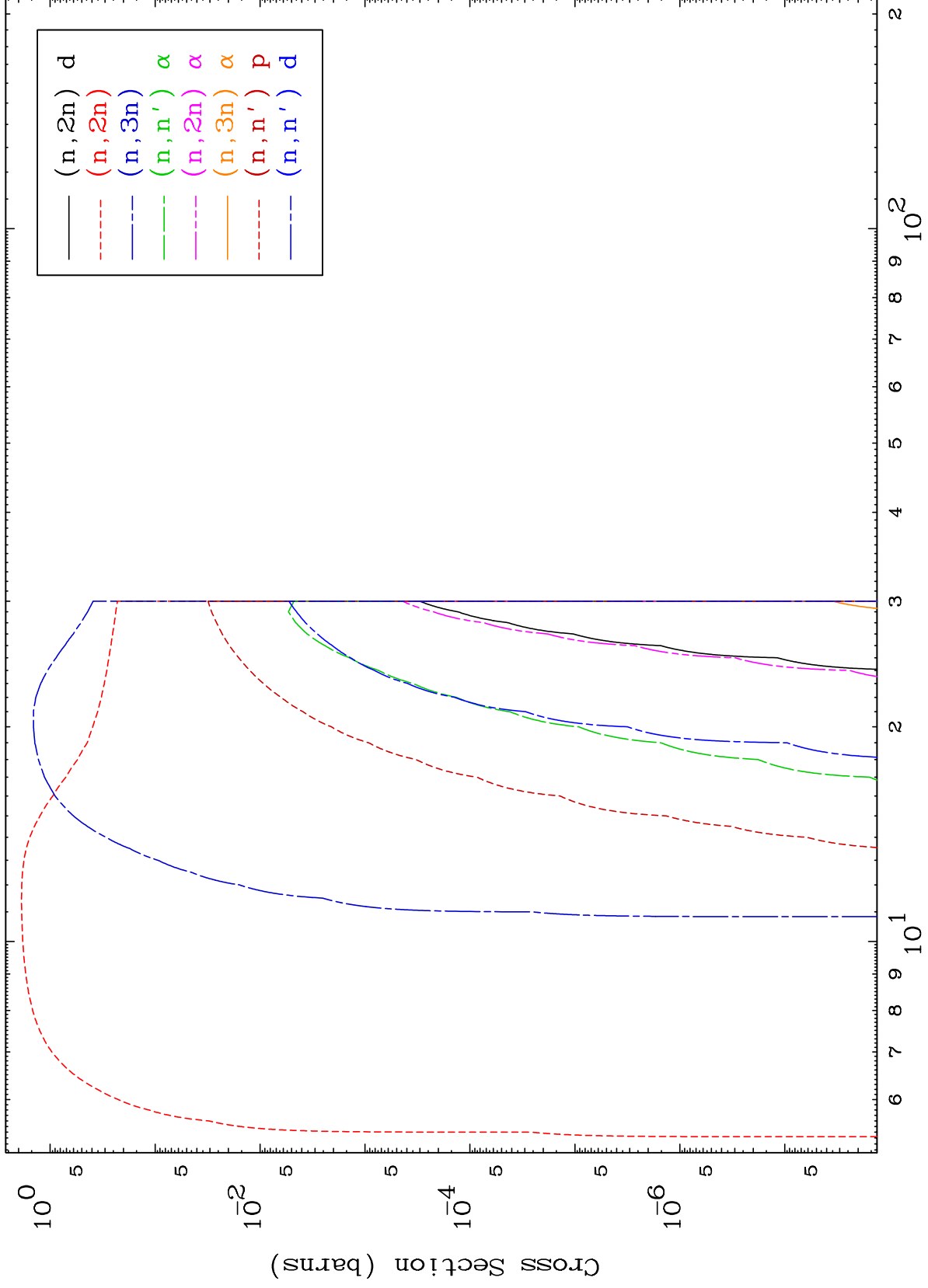
49-In-127n



MAT 4969

Neutron Absorption
293 Kelvin Cross Sections

49-In-127n



2

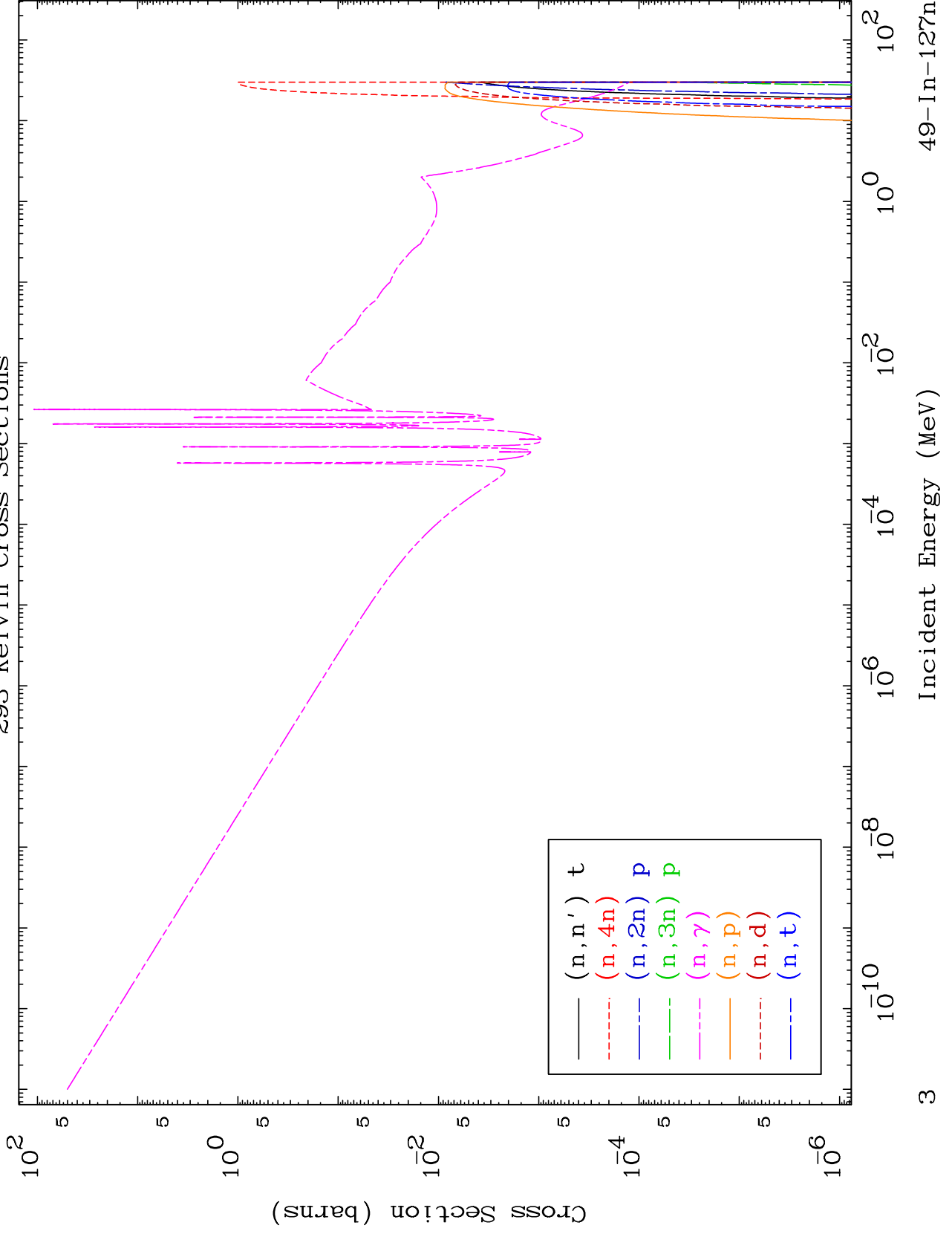
Incident Energy (MeV)

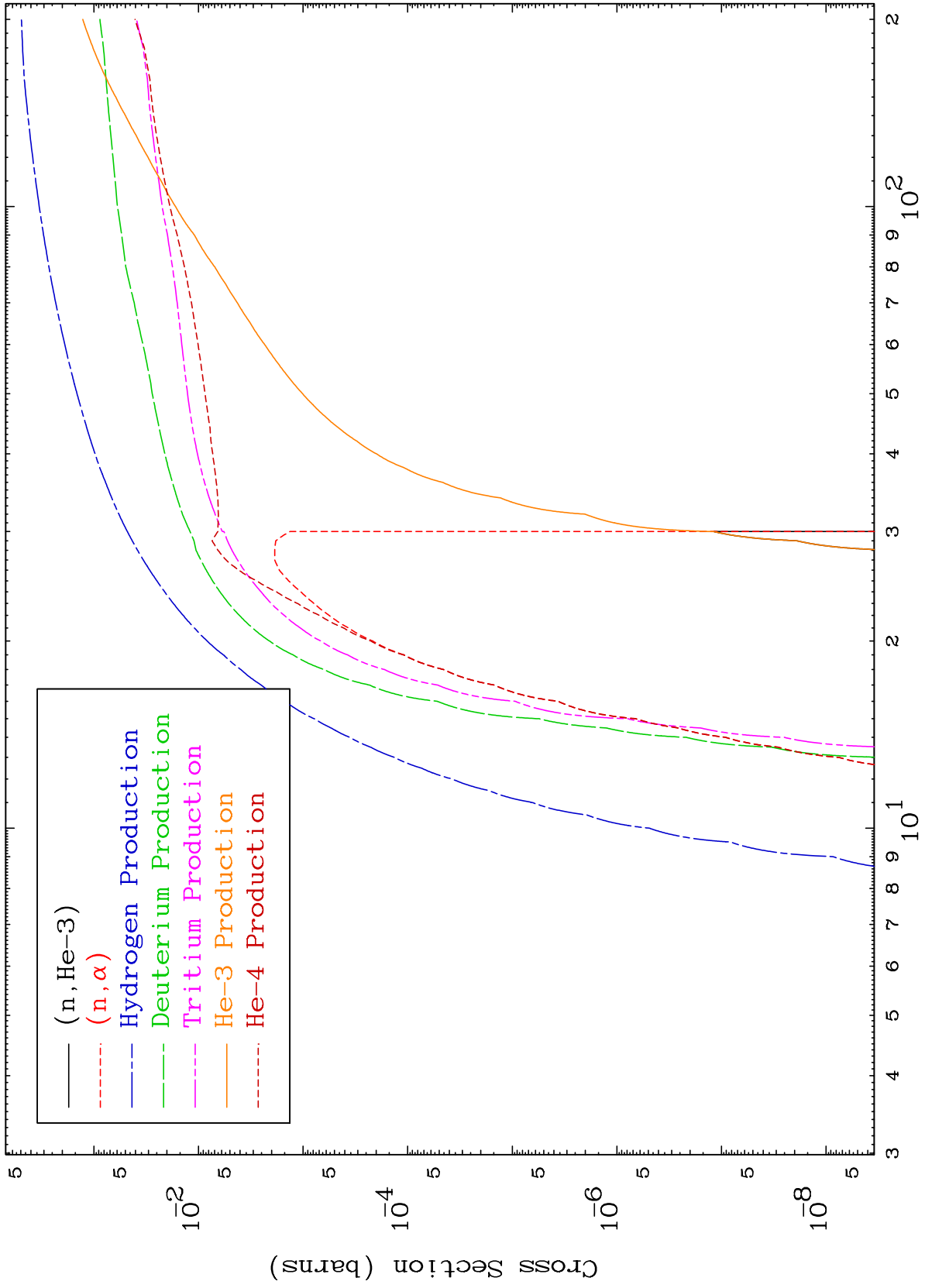
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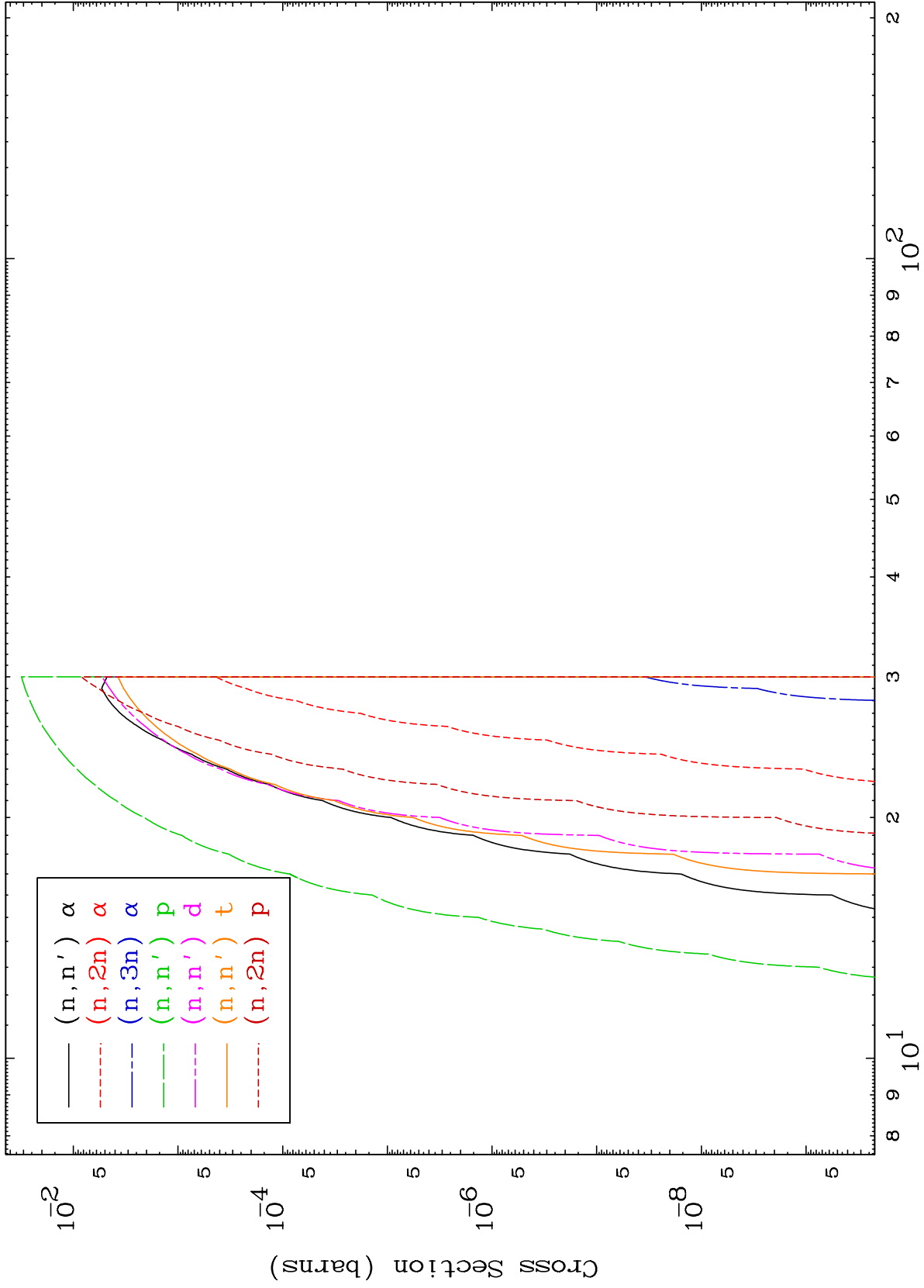
MAT 4969

Neutron Absorption
293 Kelvin Cross Sections

49-In-127n



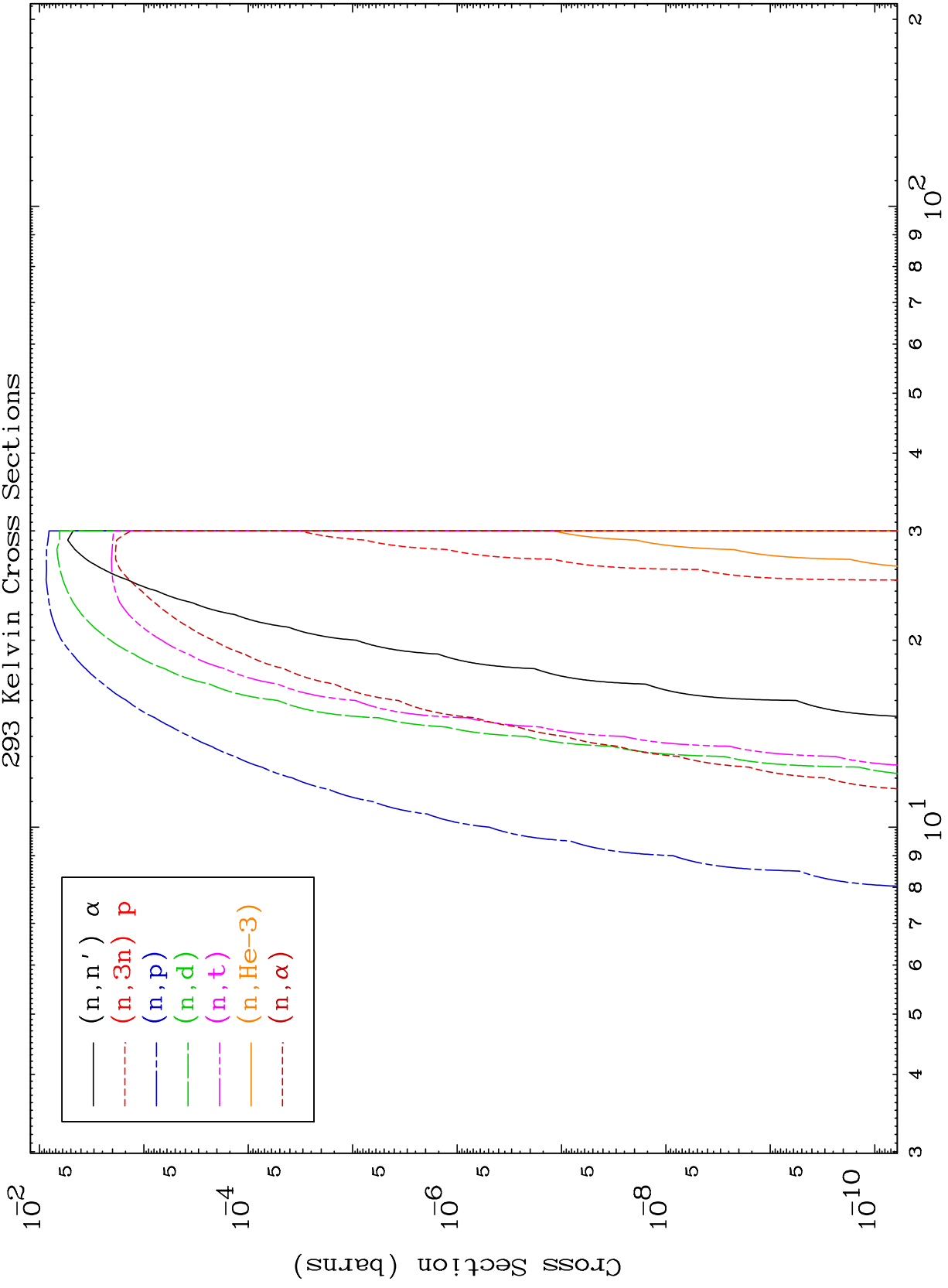




MAT 4969

Charged Particle
293 Kelvin Cross Sections

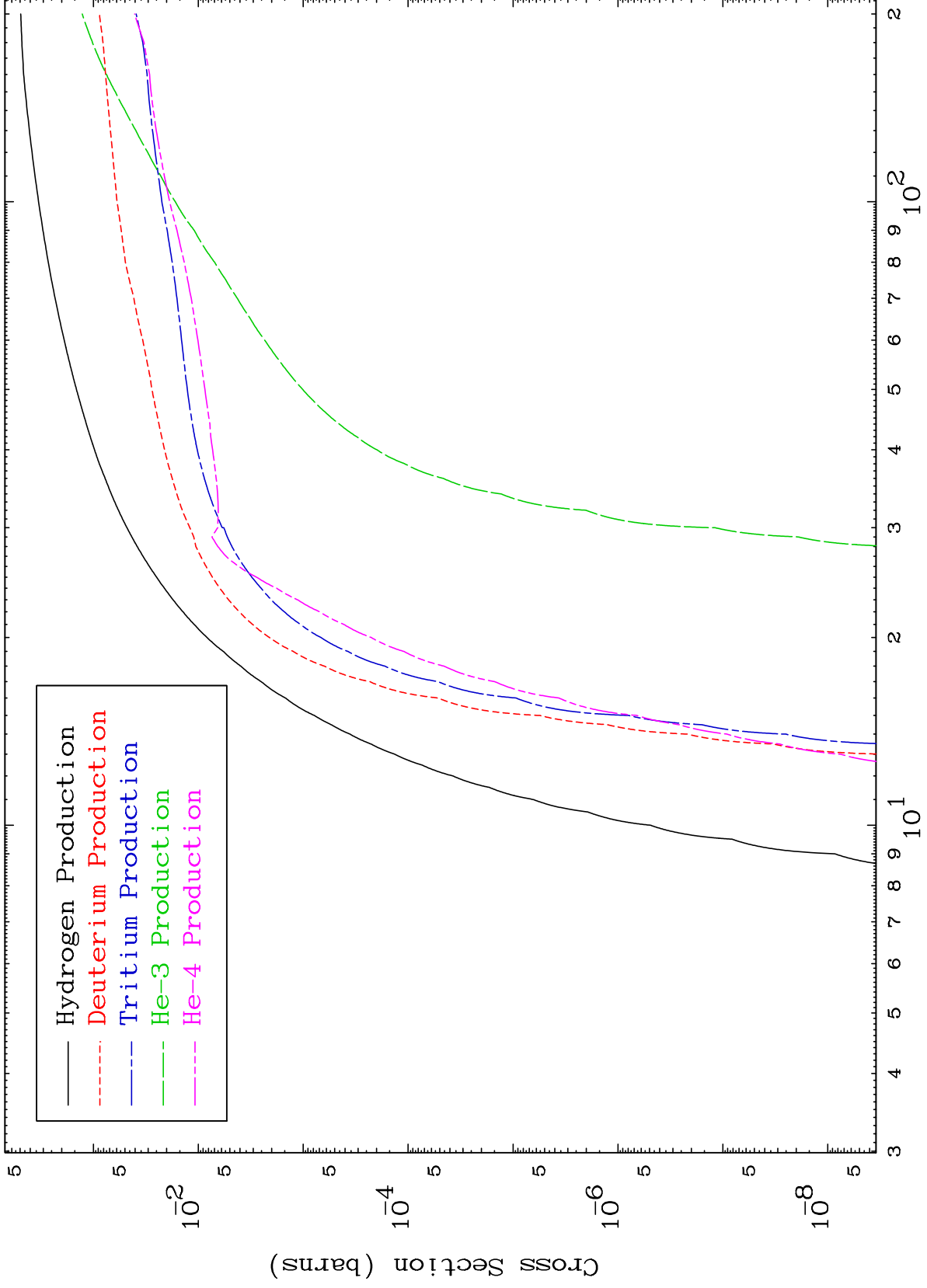
49-In-127n



MAT 4969

Particle Production
293 Kelvin Cross Sections

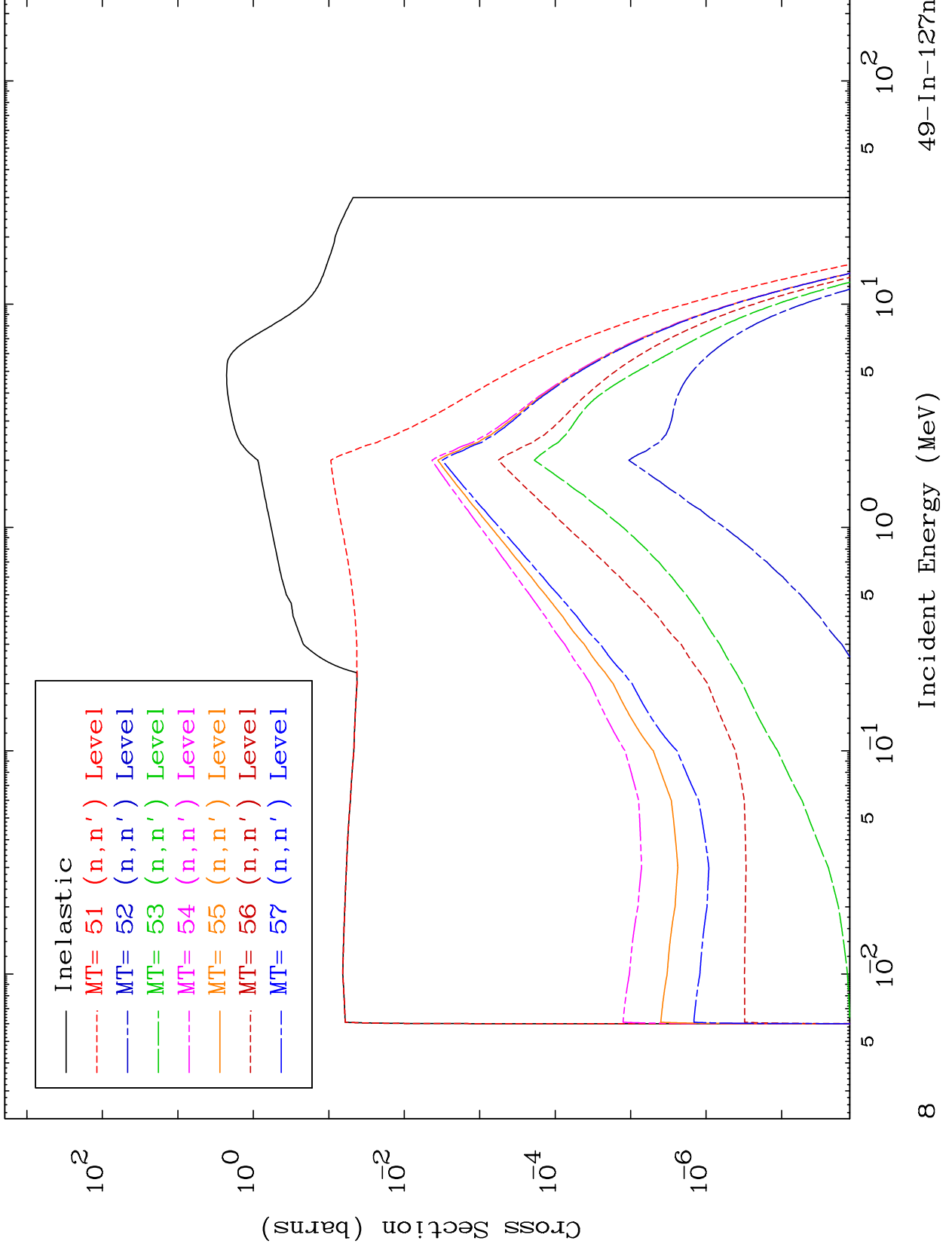
49-In-127n



MAT 4969

(n,n') Levels
293 Kelvin Cross Sections

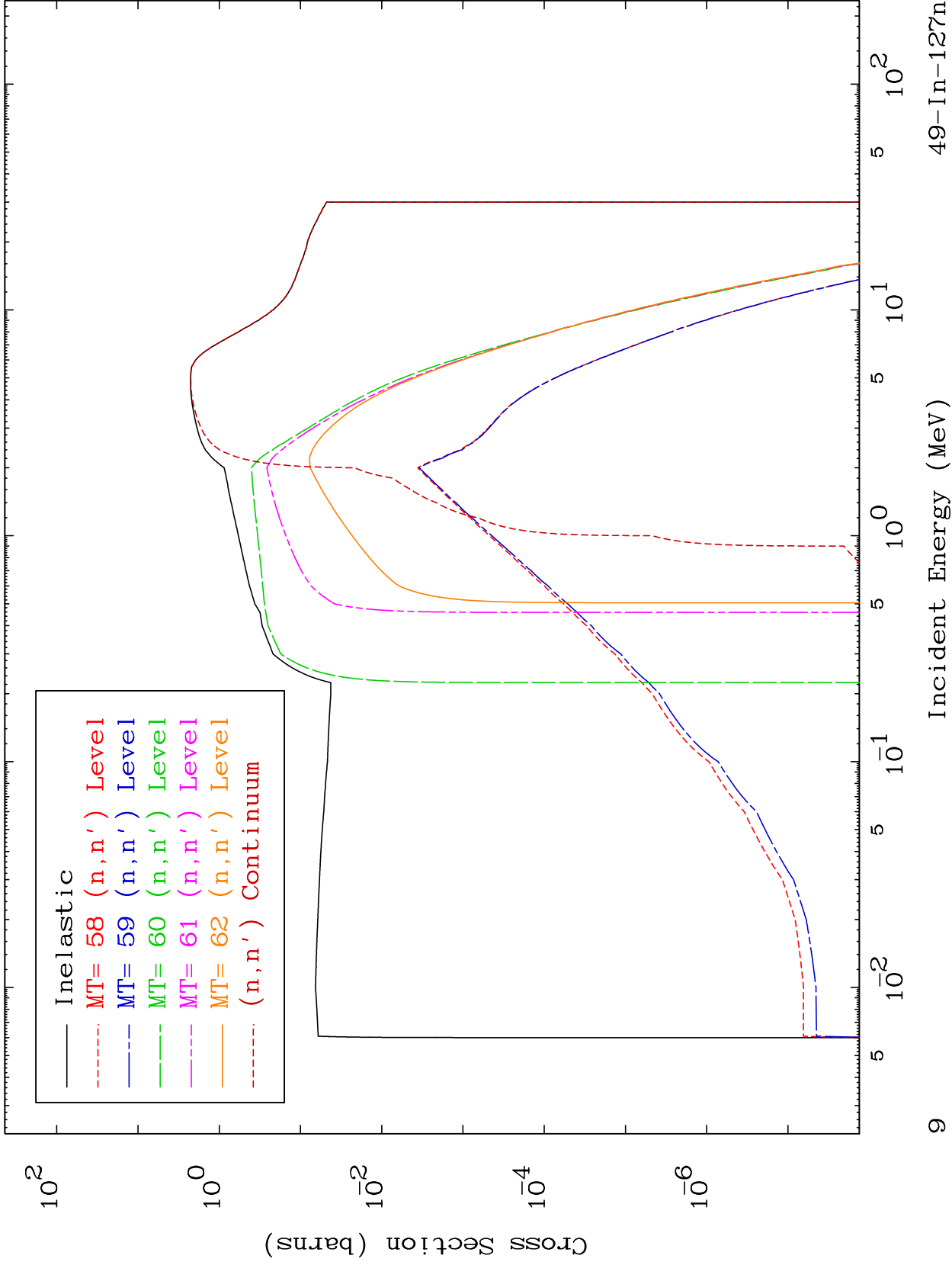
49-In-127n



MAT 4969

293 Kelvin Cross Sections
(n,n') Levels

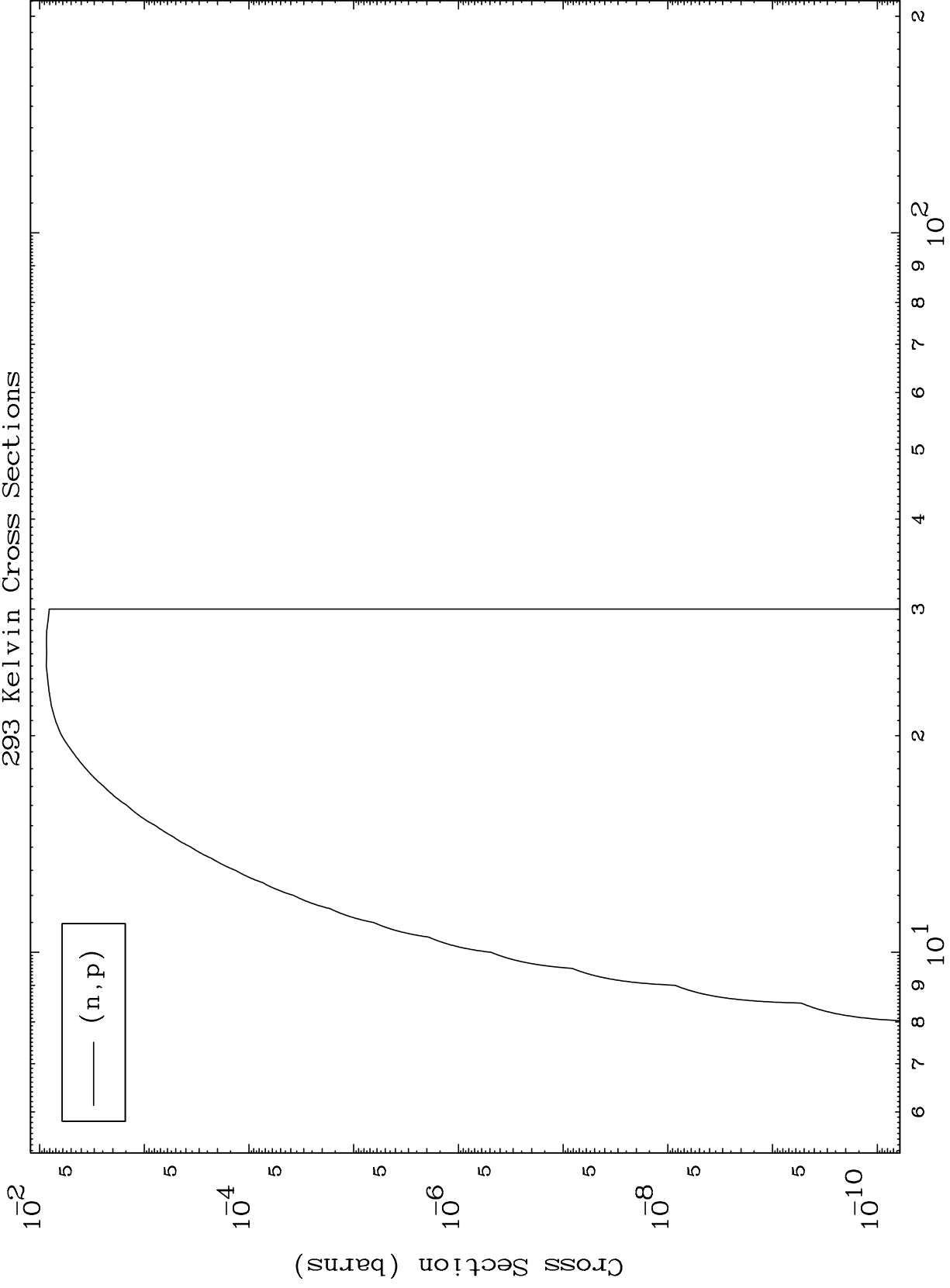
49-In-127n



MAT 4969

(n,p) Levels
293 Kelvin Cross Sections

49-In-127n



10

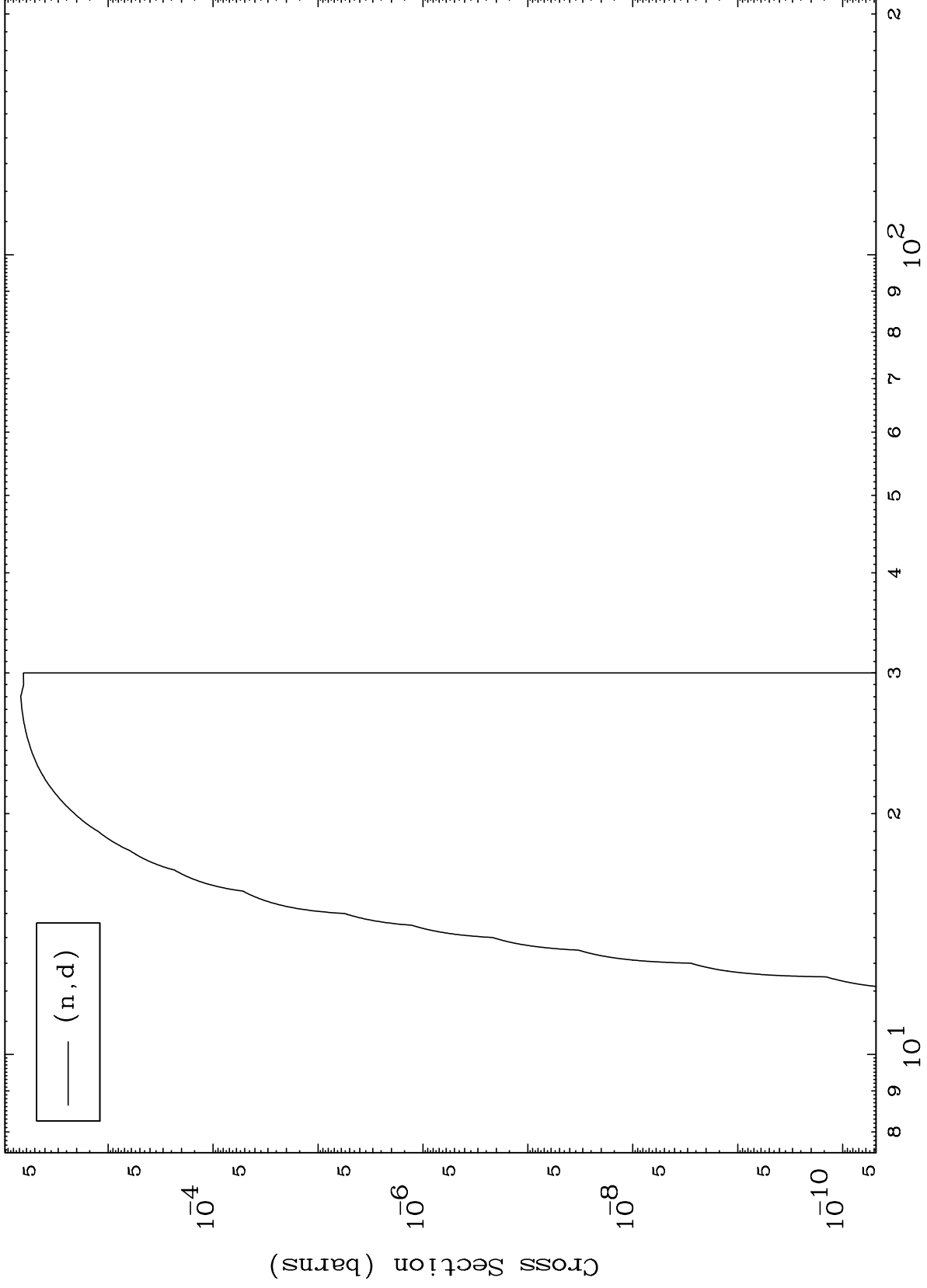
Incident Energy (MeV)

49-In-127n

MAT 4969

(n,d) Levels
293 Kelvin Cross Sections

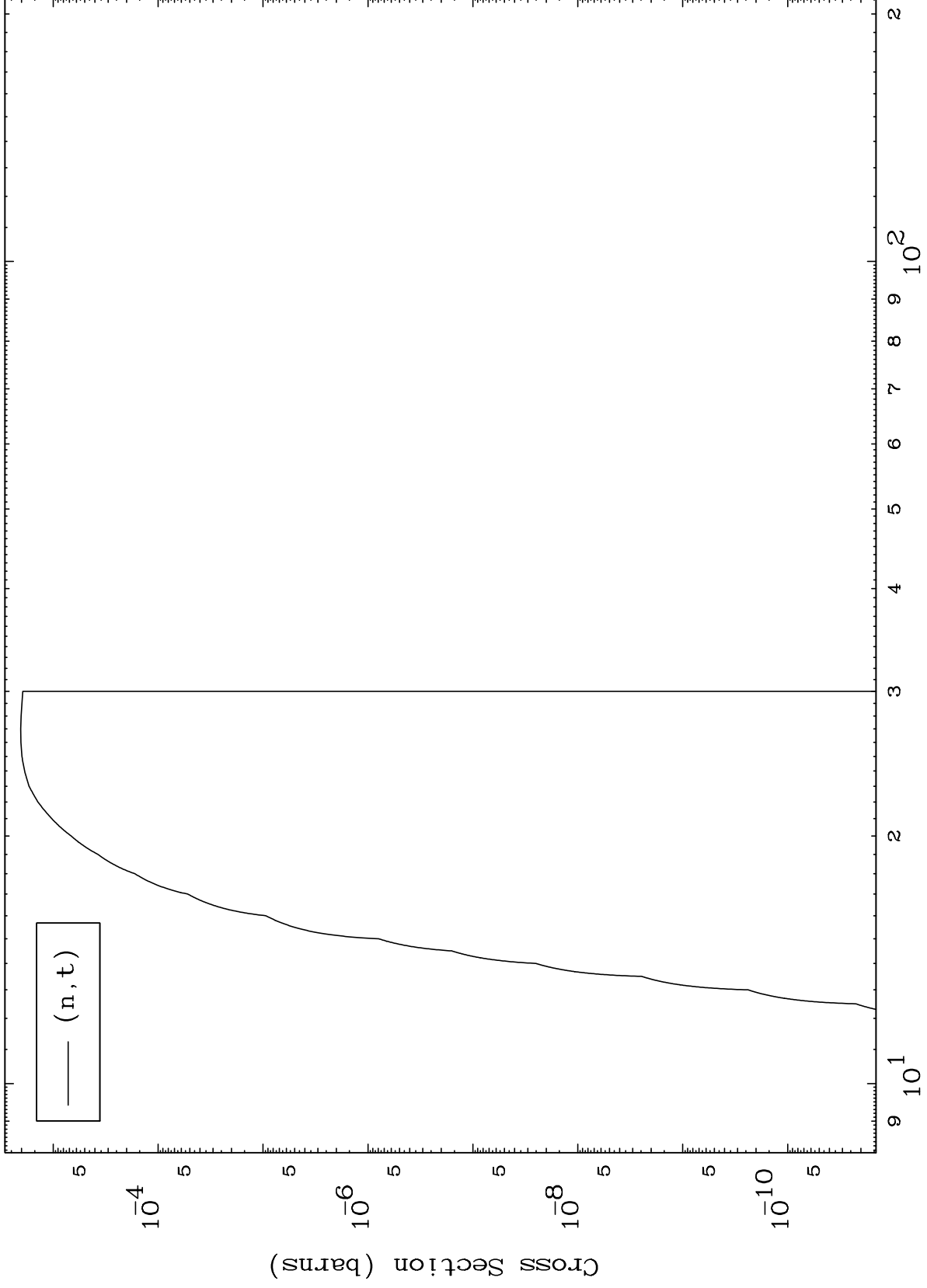
49-In-127n



MAT 4969

(n,t) Levels
293 Kelvin Cross Sections

49-In-127n



12

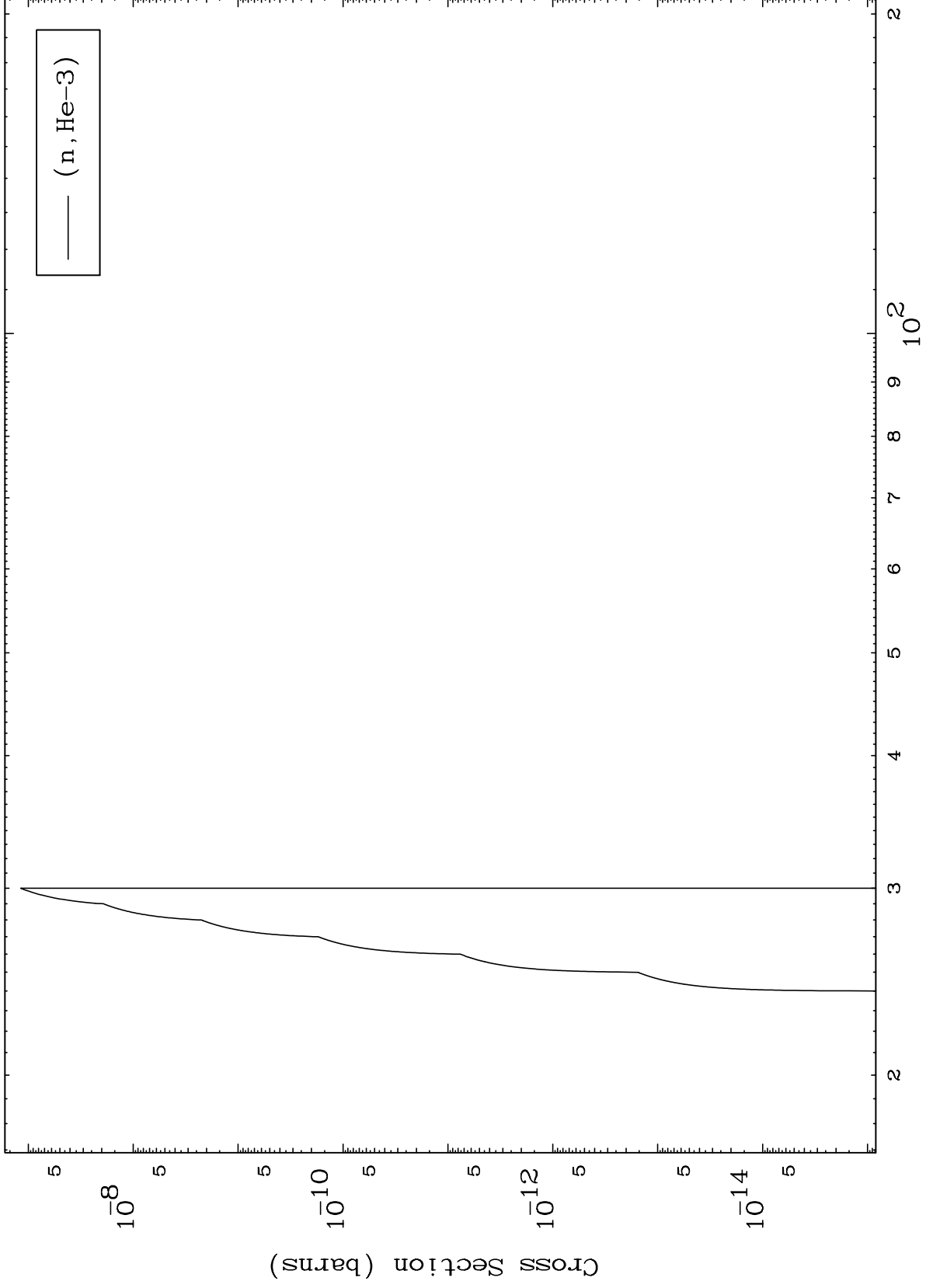
Incident Energy (MeV)

49-In-127n

MAT 4969

(n,He3) Levels
293 Kelvin Cross Sections

49-In-127n



13

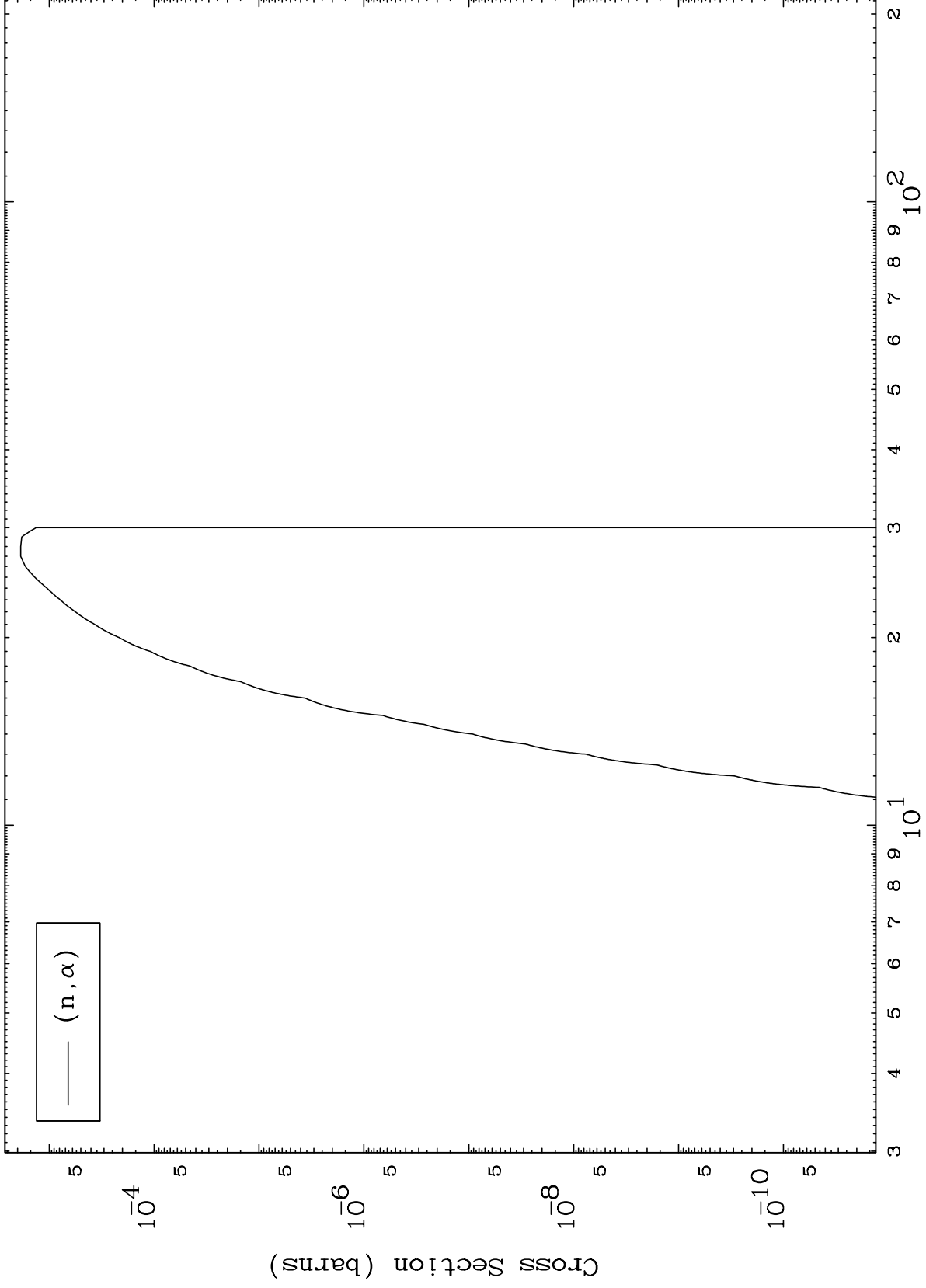
Incident Energy (MeV)

49-In-127n

MAT 4969

(n, α) Levels
293 Kelvin Cross Sections

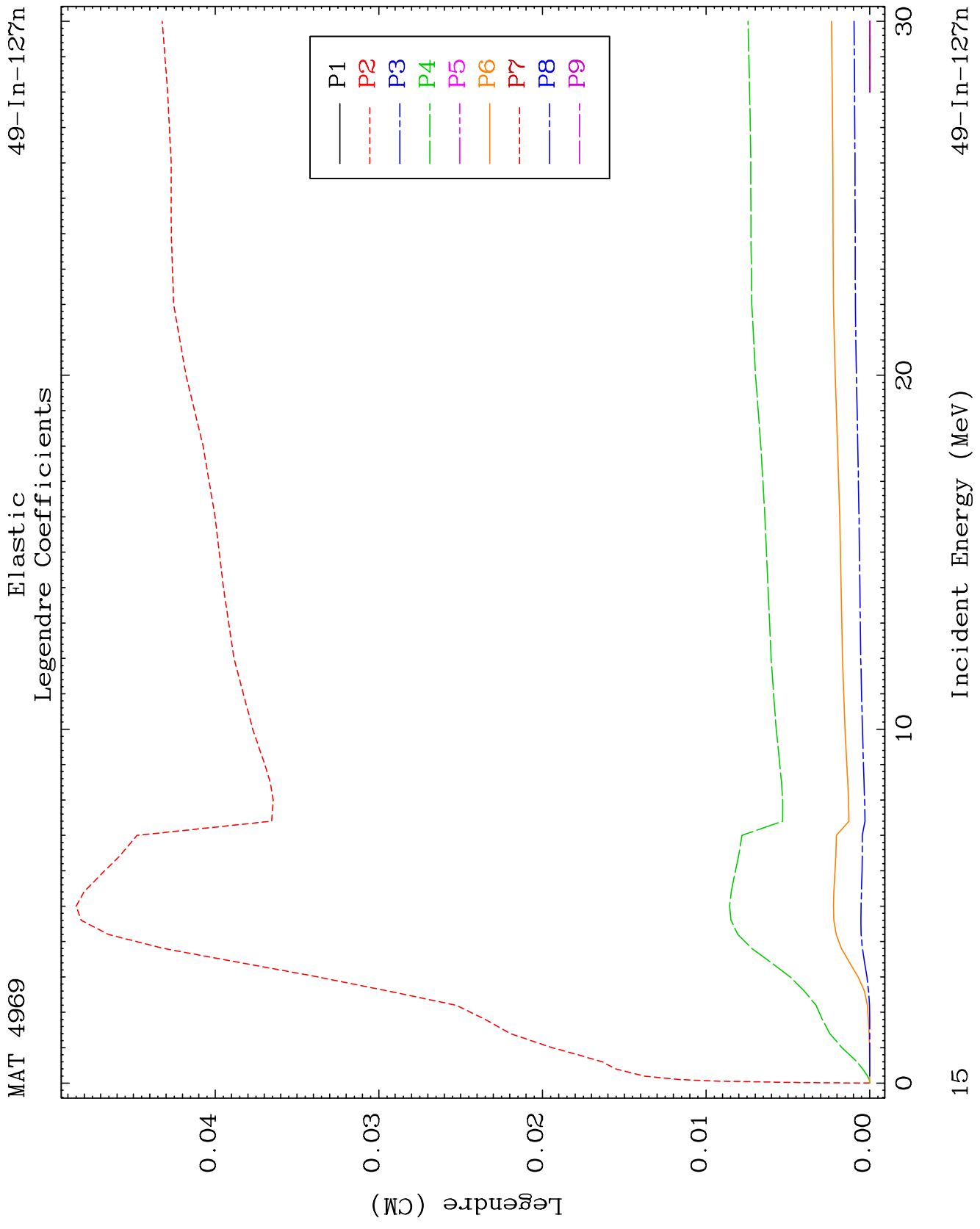
49-In-127n



14

Incident Energy (MeV)

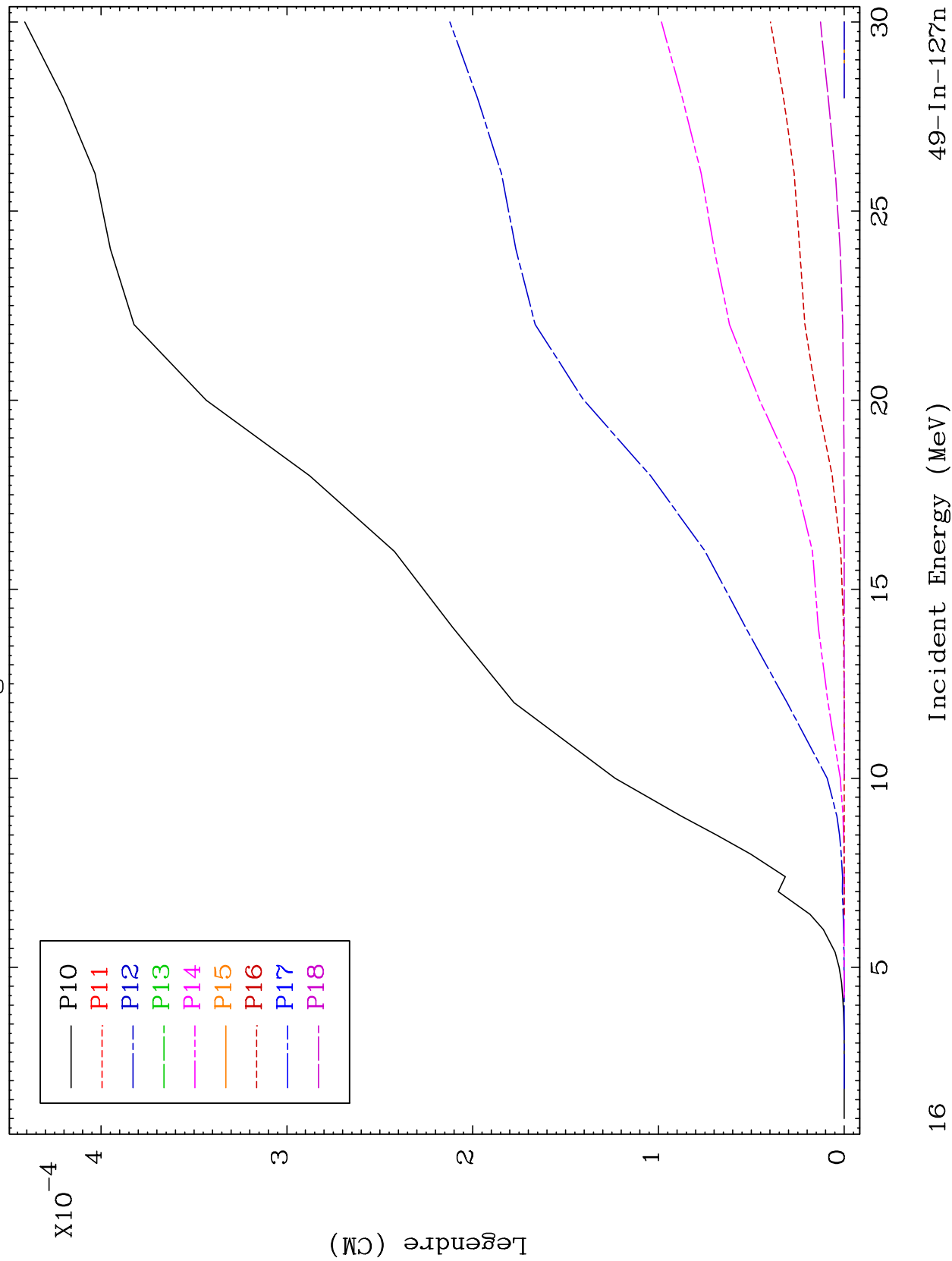
49-In-127n



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Elastic Legendre Coefficients

49-In-127n

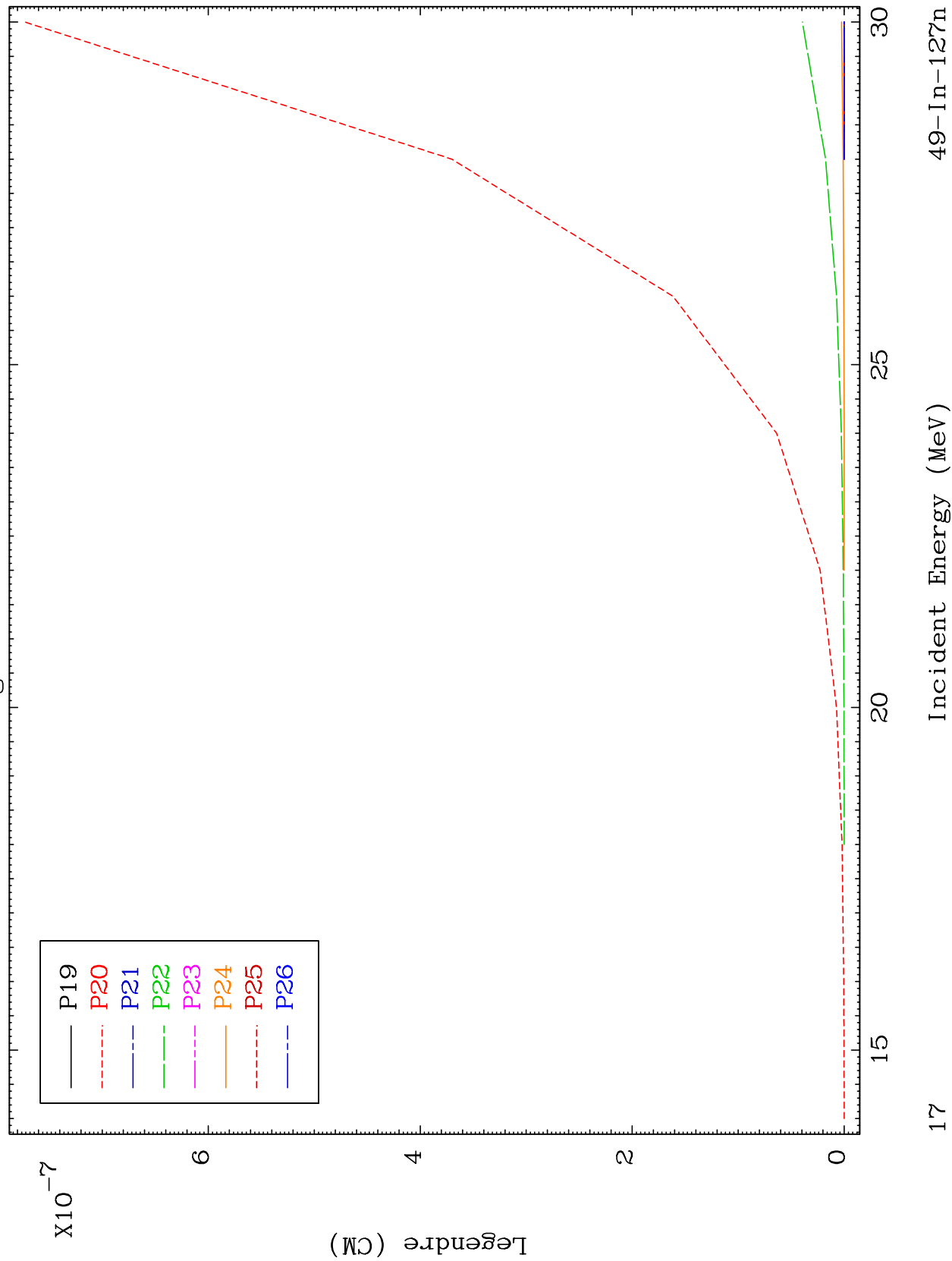


- P10
- P11
- P12
- P13
- P14
- P15
- P16
- P17
- P18

MAT 4969

Elastic Legendre Coefficients

49-In-127n



17

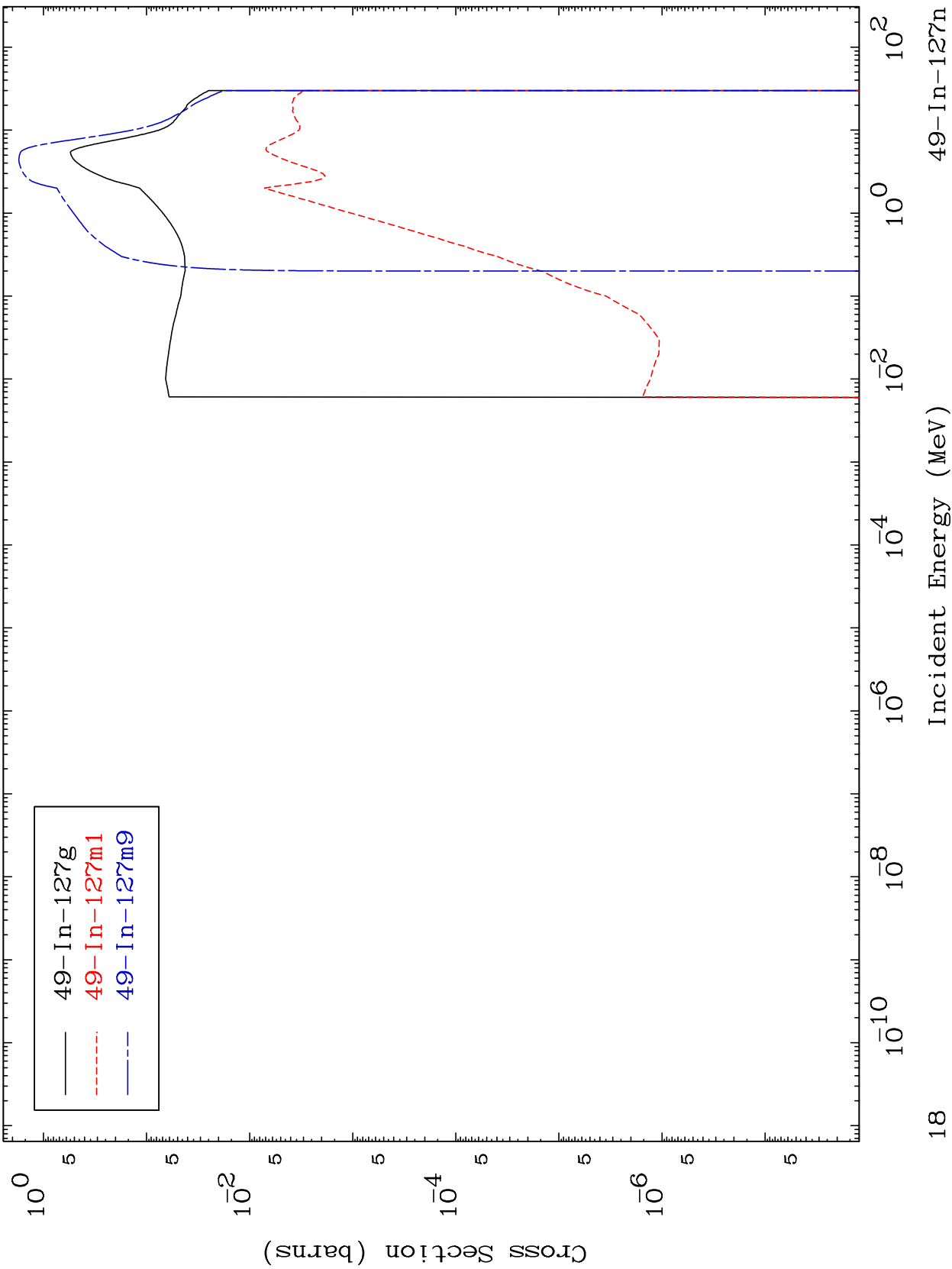
Incident Energy (MeV)

49-In-127n

MAT 4969

49-In-127n

Inelastic
Radionuclide Production Cross Section

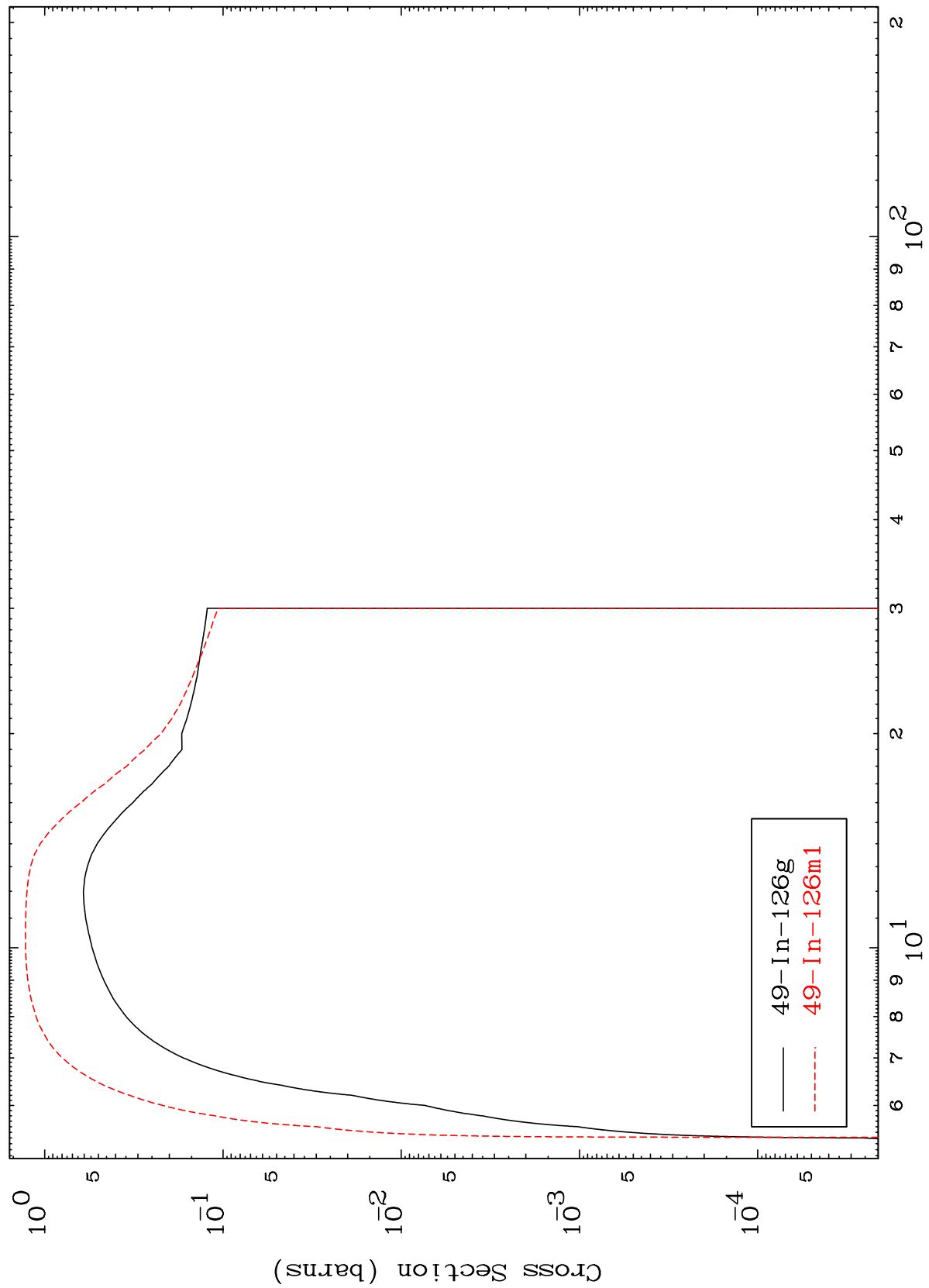


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(n,2n)

49-In-127n

Radionuclide Production Cross Section



19

Incident Energy (MeV)

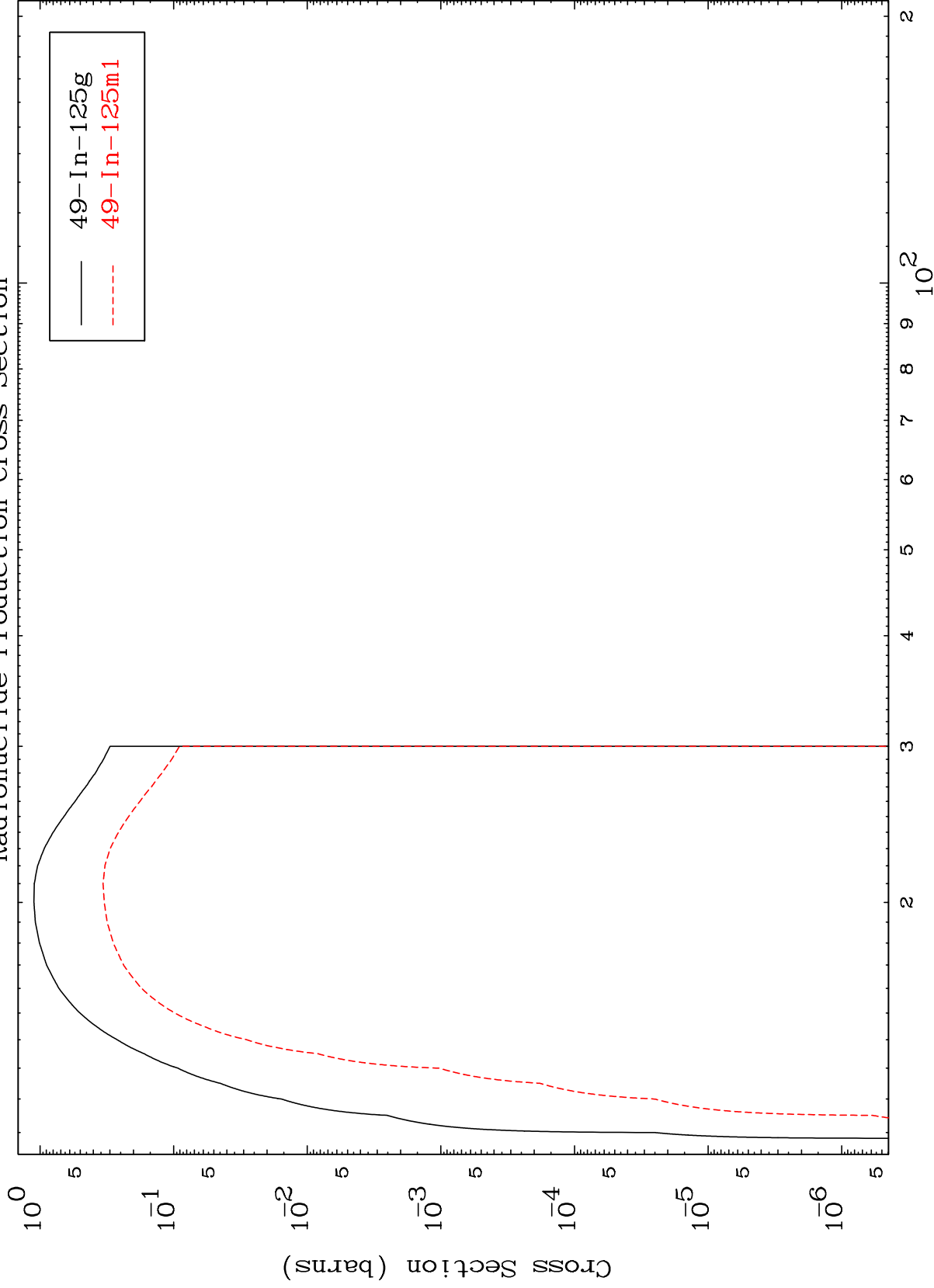
49-In-127n

MAT 4969

(n,3n)

49-In-127n

Radionuclide Production Cross Section



Incident Energy (MeV)

49-In-127n

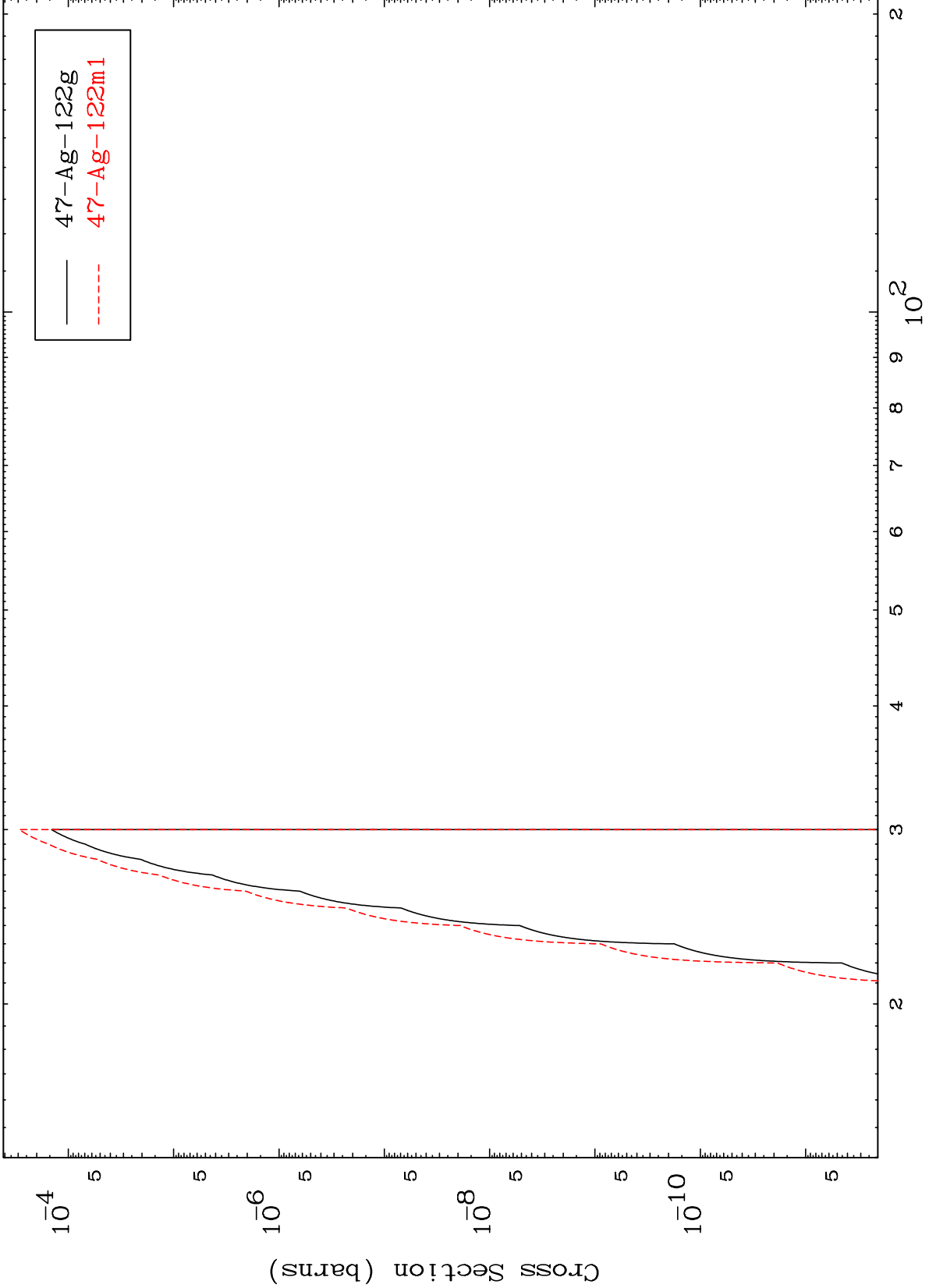
20

MAT 4969

(n,2n) α

49-In-127n

Radionuclide Production Cross Section



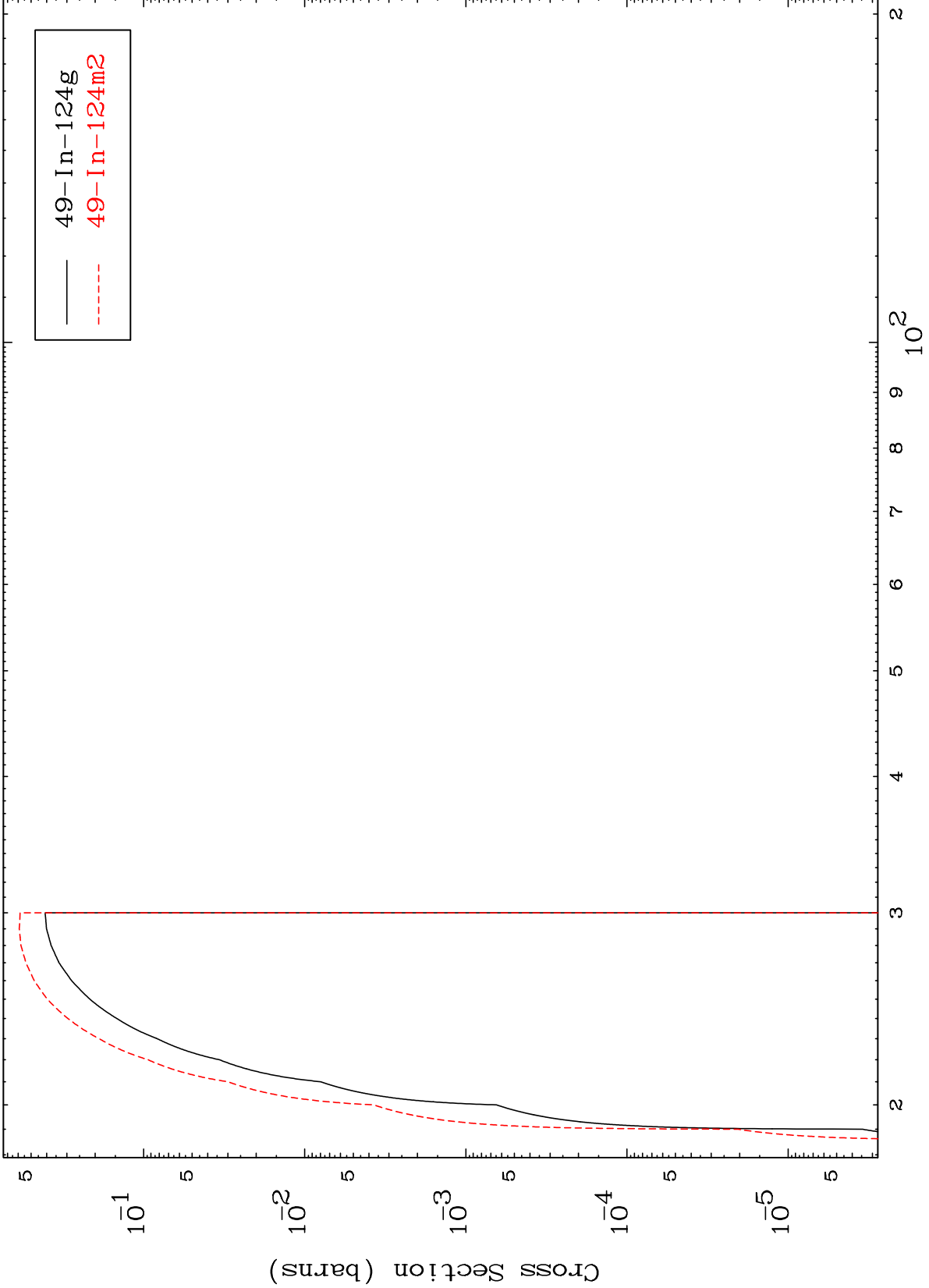
47-Ag-122g
47-Ag-122m1

MAT 4969

(n,4n)

49-In-127n

Radionuclide Production Cross Section



22

Incident Energy (MeV)

49-In-127n

MAT 4969

$^{49}\text{In}-^{127}\text{n}$

Radionuclide Production Cross Section
(n, γ)

