

Program EVALPLOT
(Version 2021-1)

by

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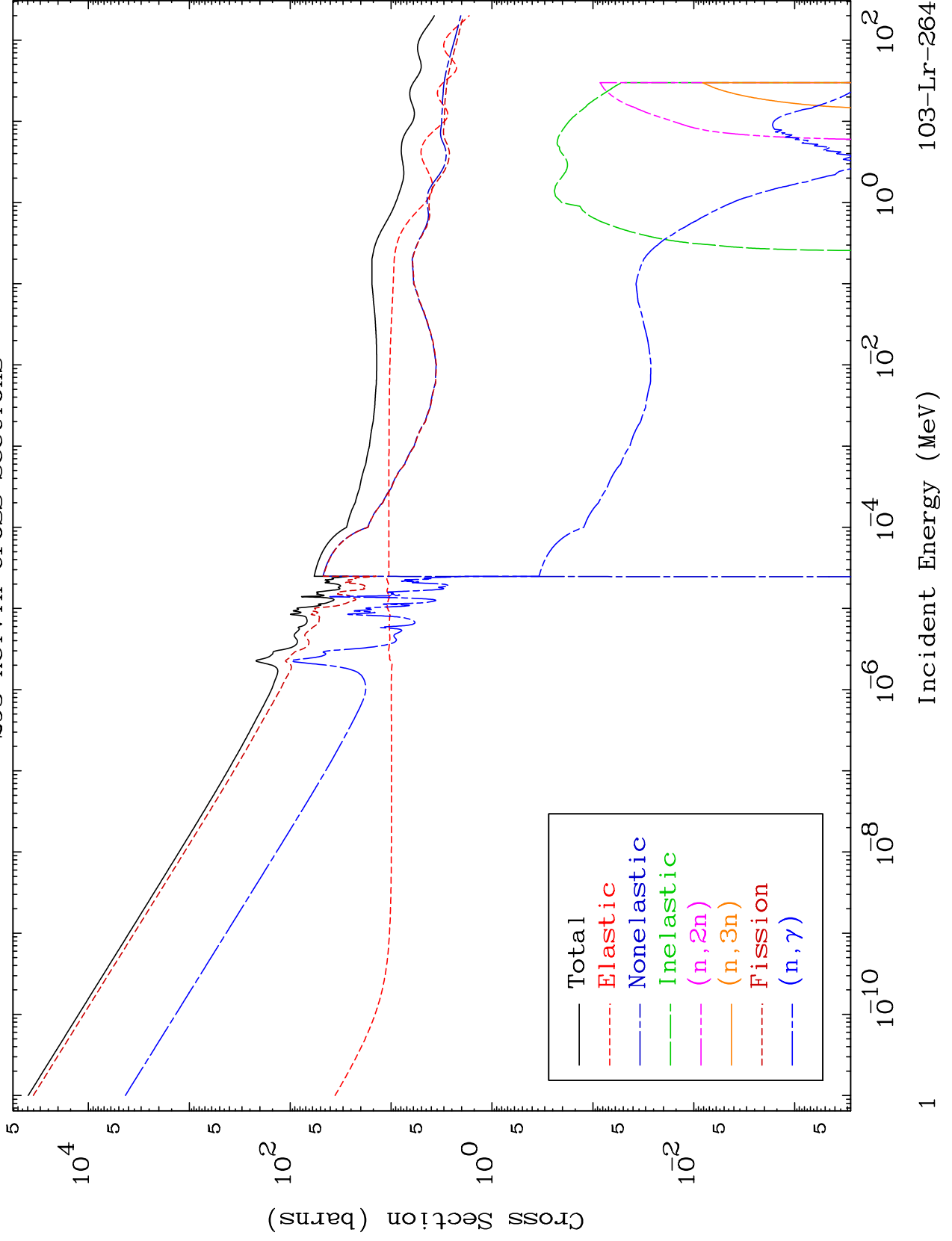
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9993

Neutron Major
293 Kelvin Cross Sections

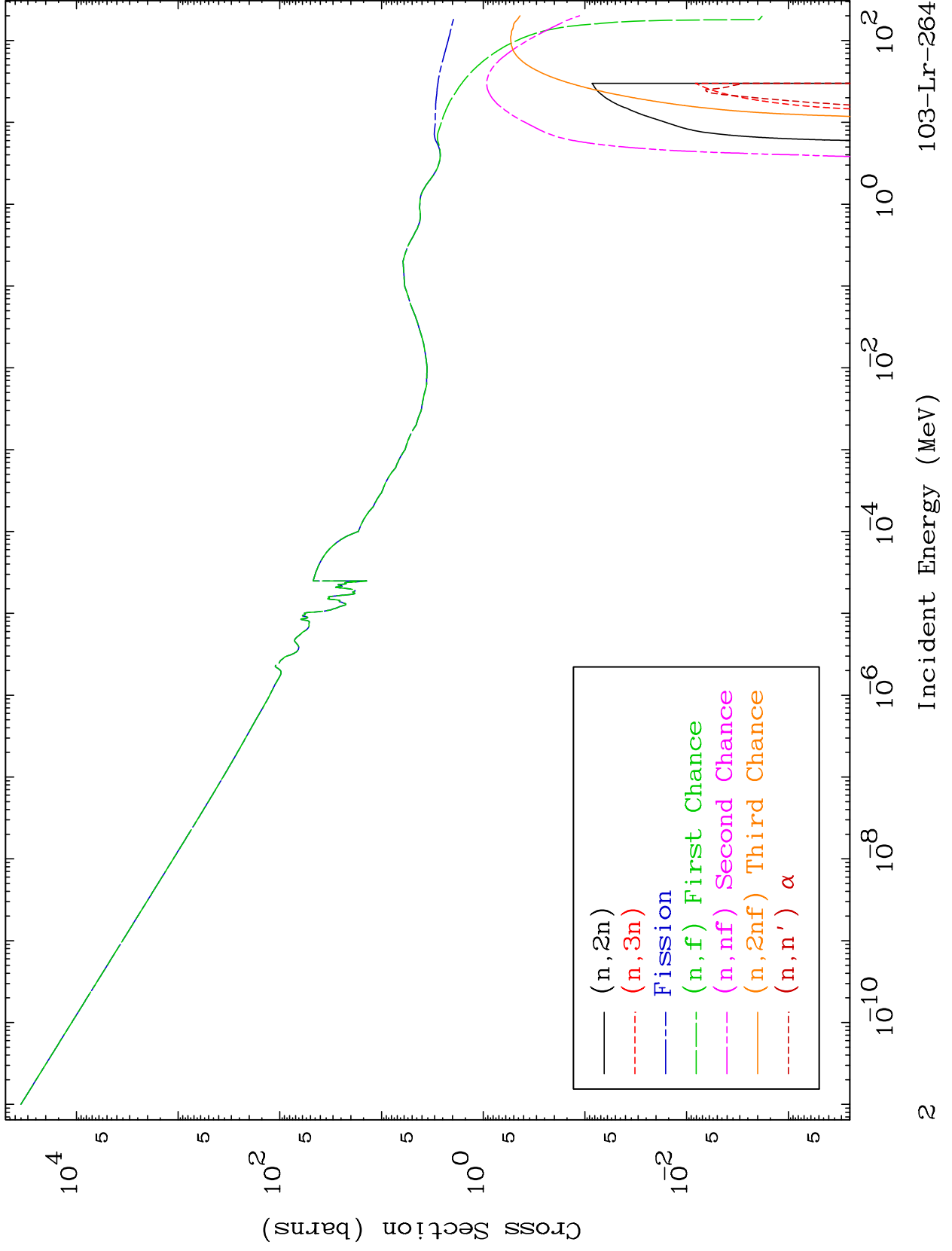
103-Lr-264



MAT 9993

Neutron Absorption
293 Kelvin Cross Sections

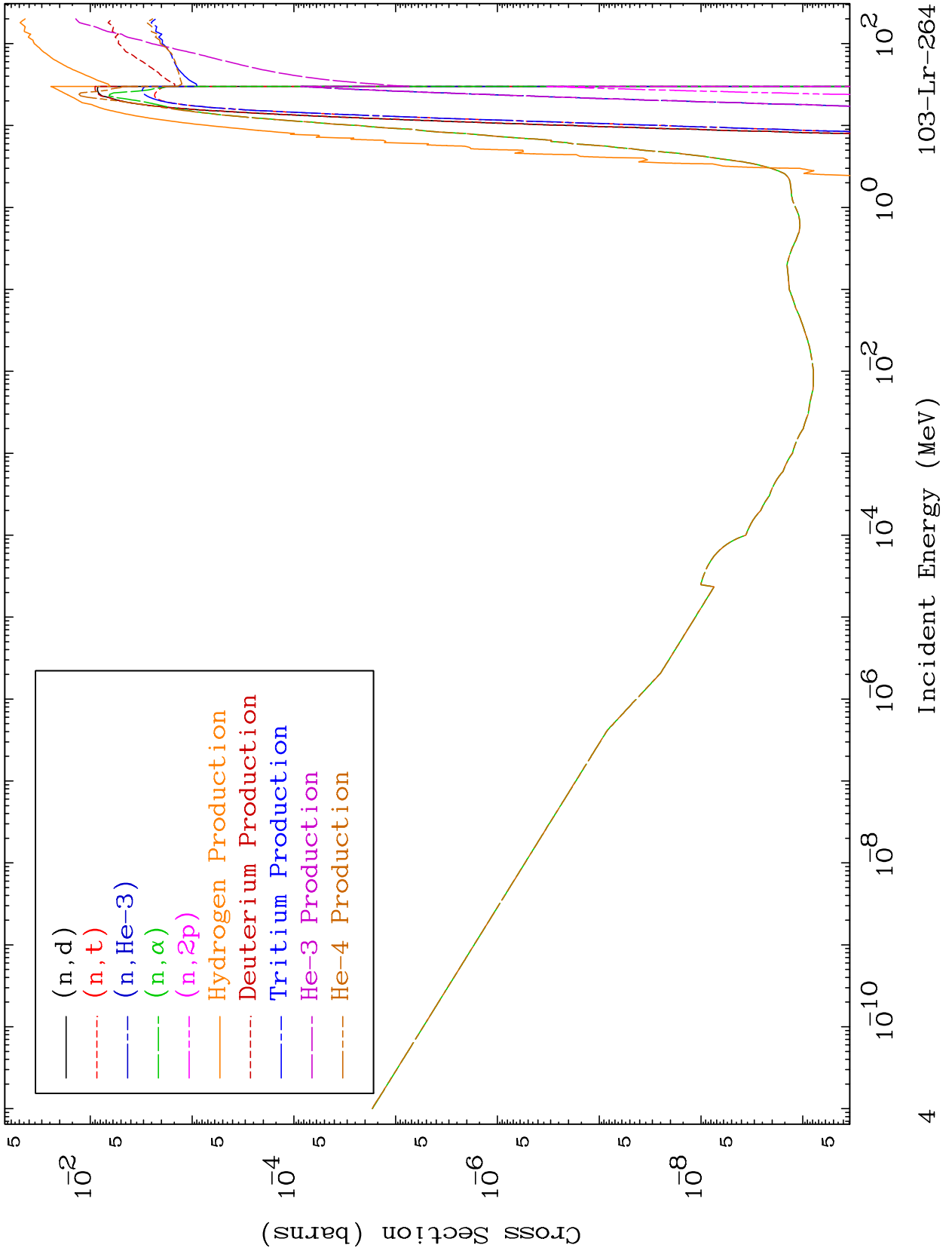
103-Lr-264



MAT 9993

Neutron Absorption
293 Kelvin Cross Sections

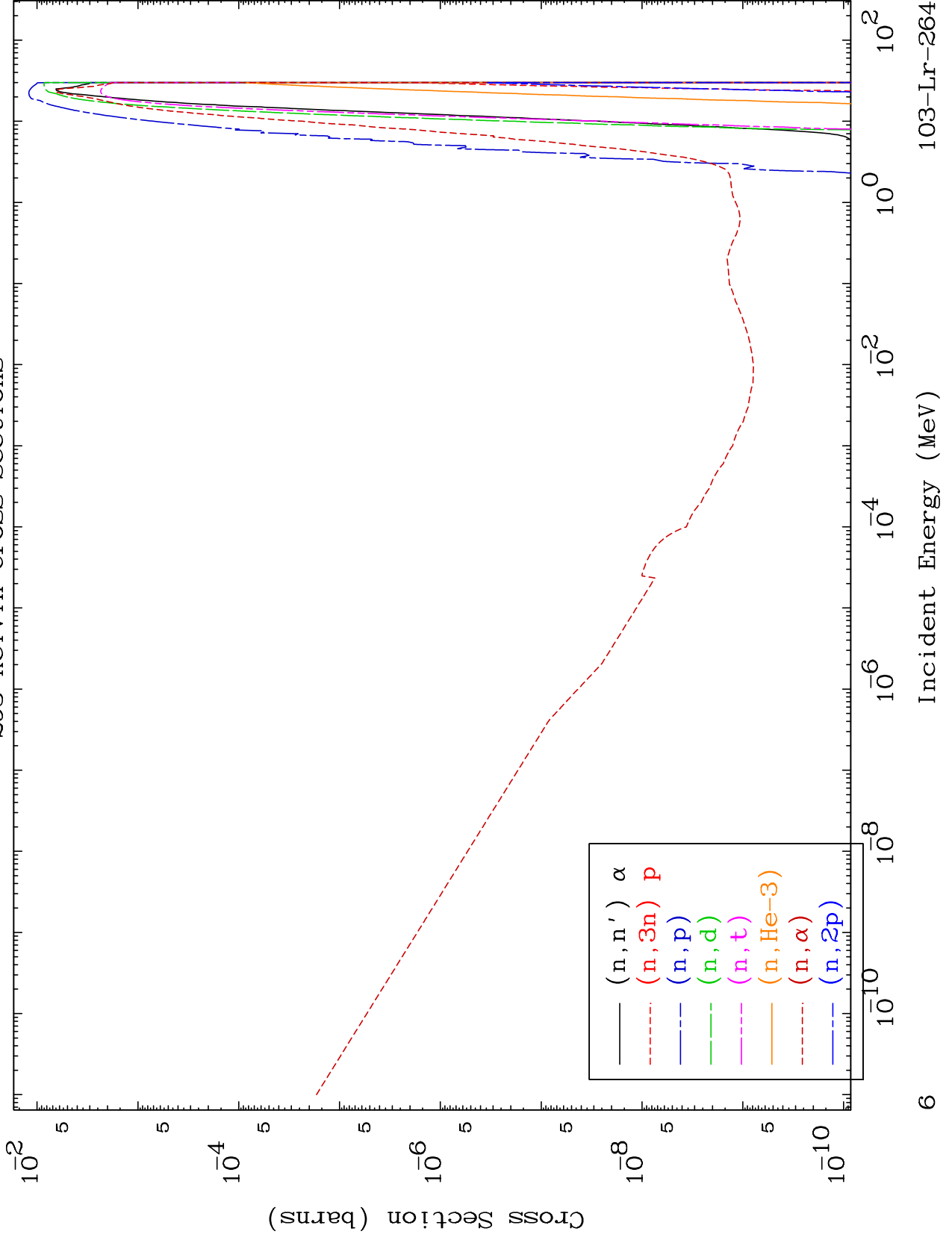
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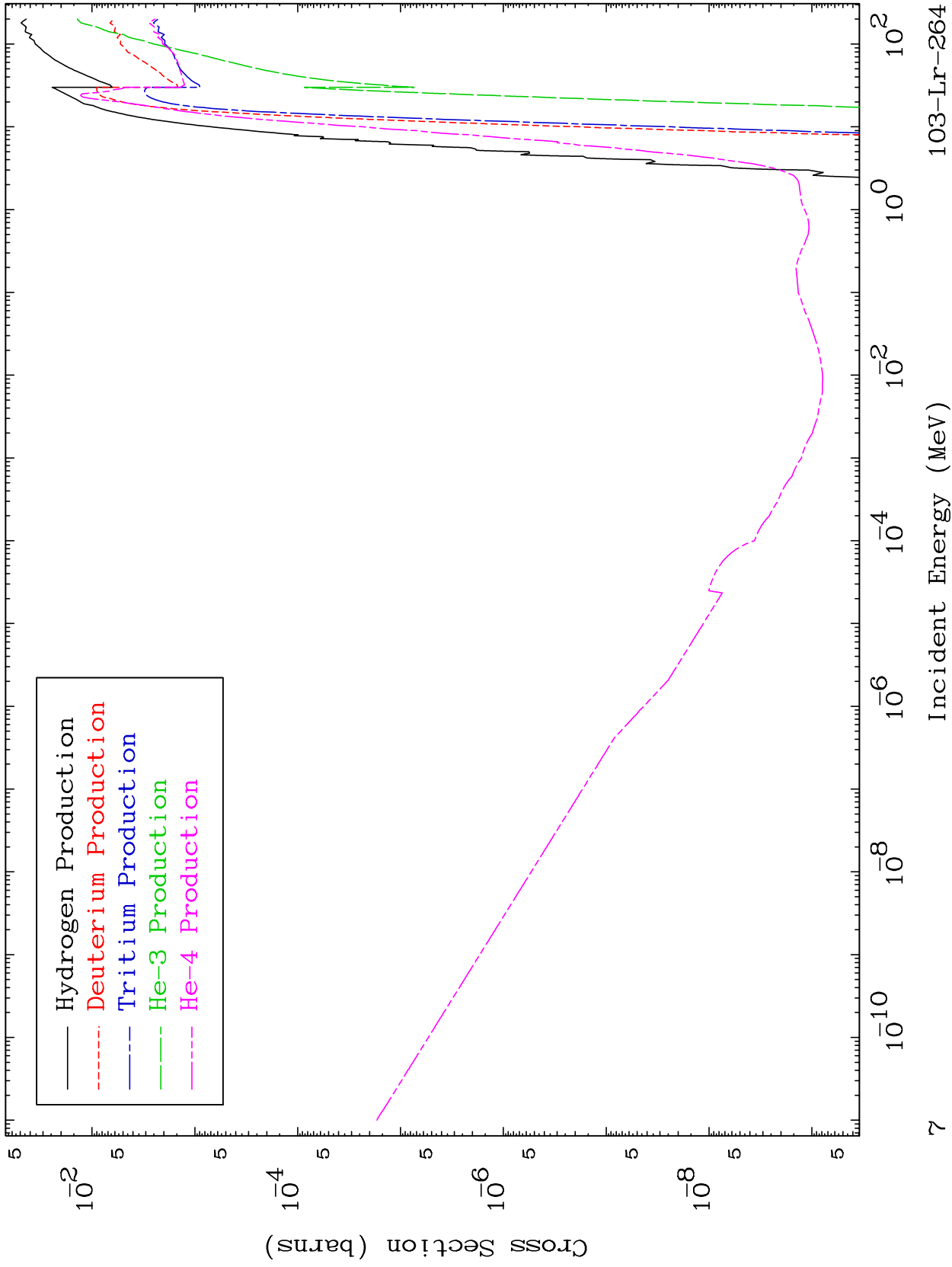


MAT 9993

Charged Particle
293 Kelvin Cross Sections

103-Lr-264

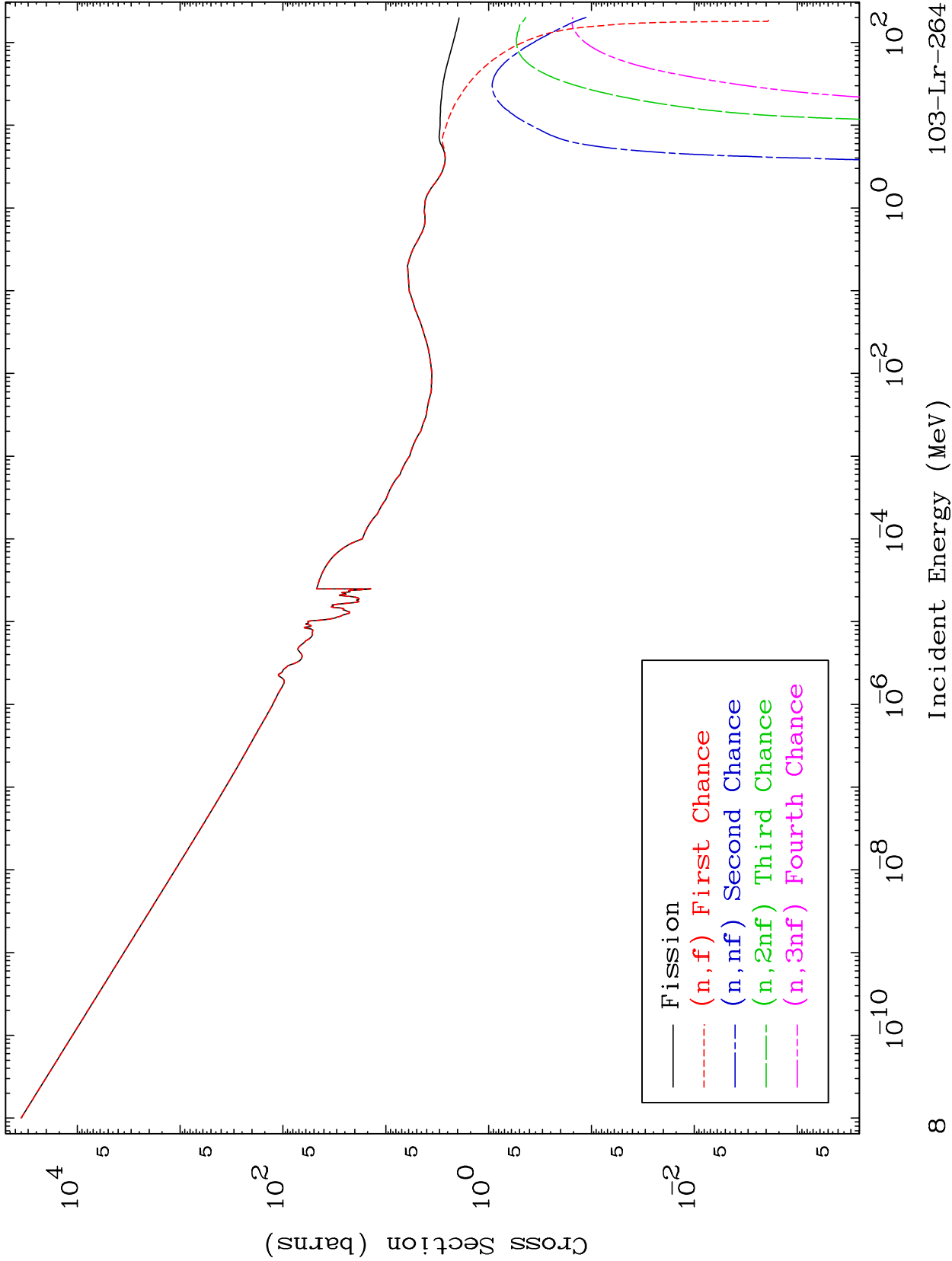




MAT 9993

Fission
293 Kelvin Cross Sections

103-Lr-264

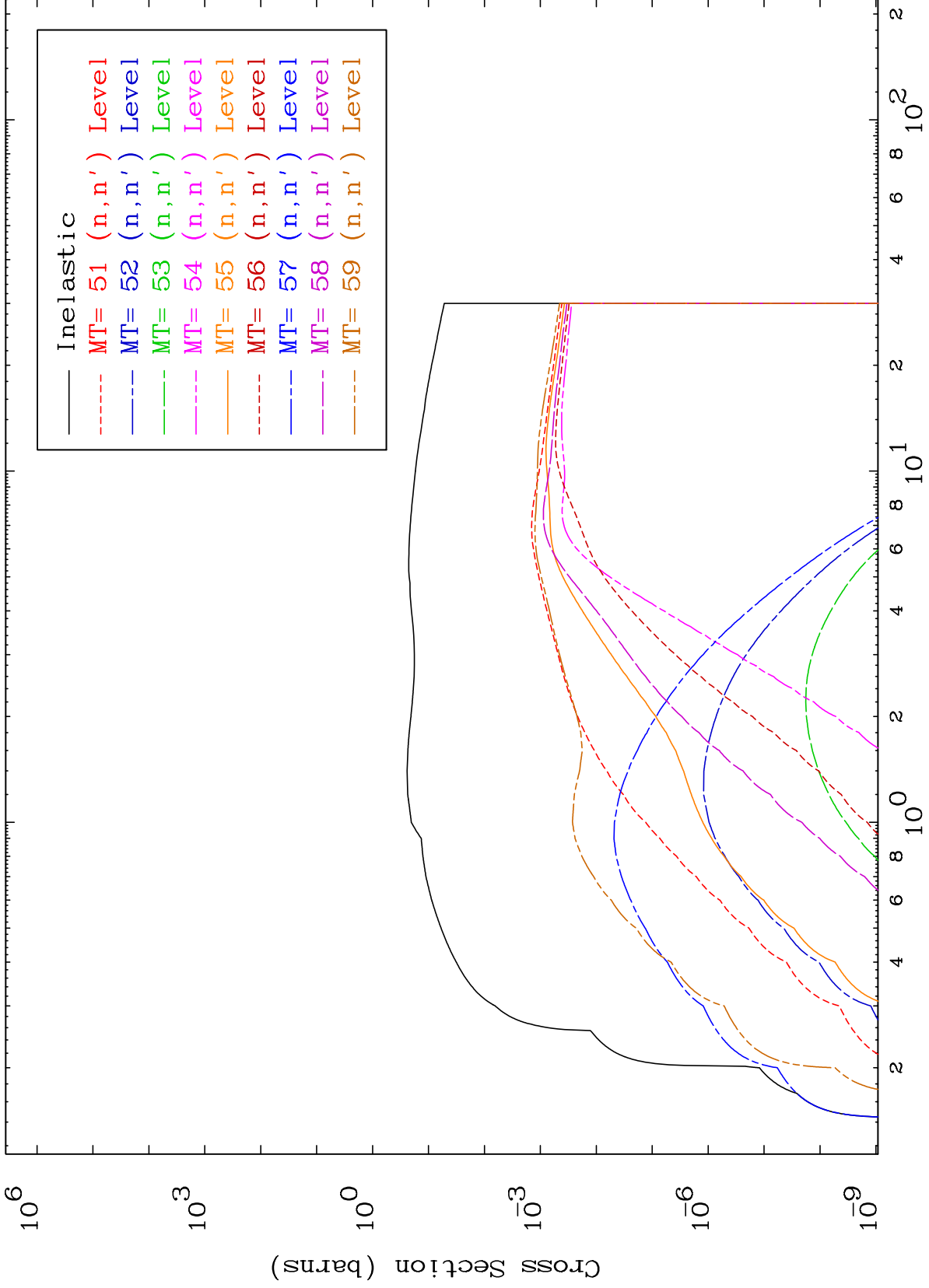


MAT 9993

(n,n') Levels

103-Lr-264

293 Kelvin Cross Sections

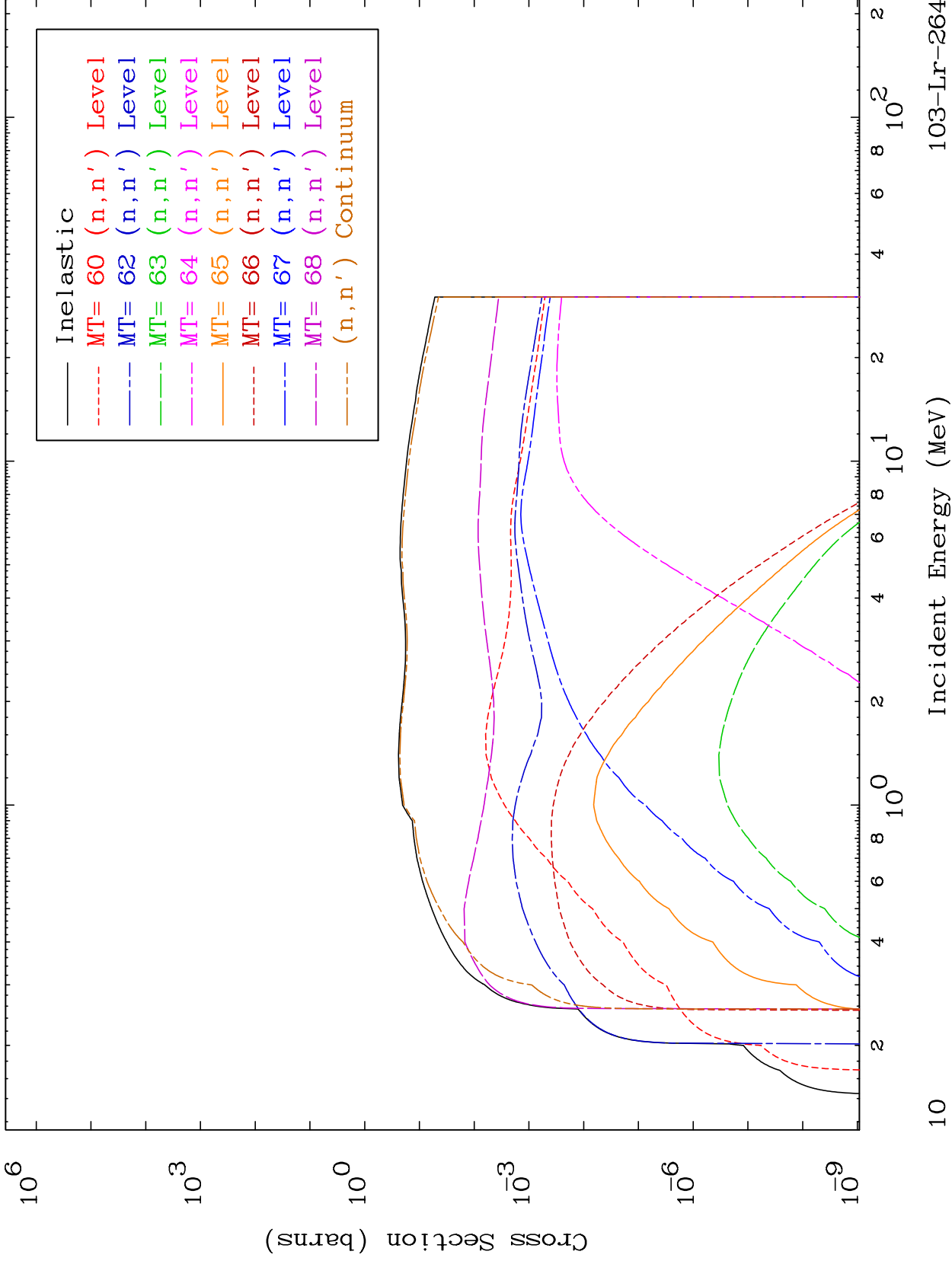


MAT 9993

(n,n') Levels

103-Lr-264

293 Kelvin Cross Sections



10

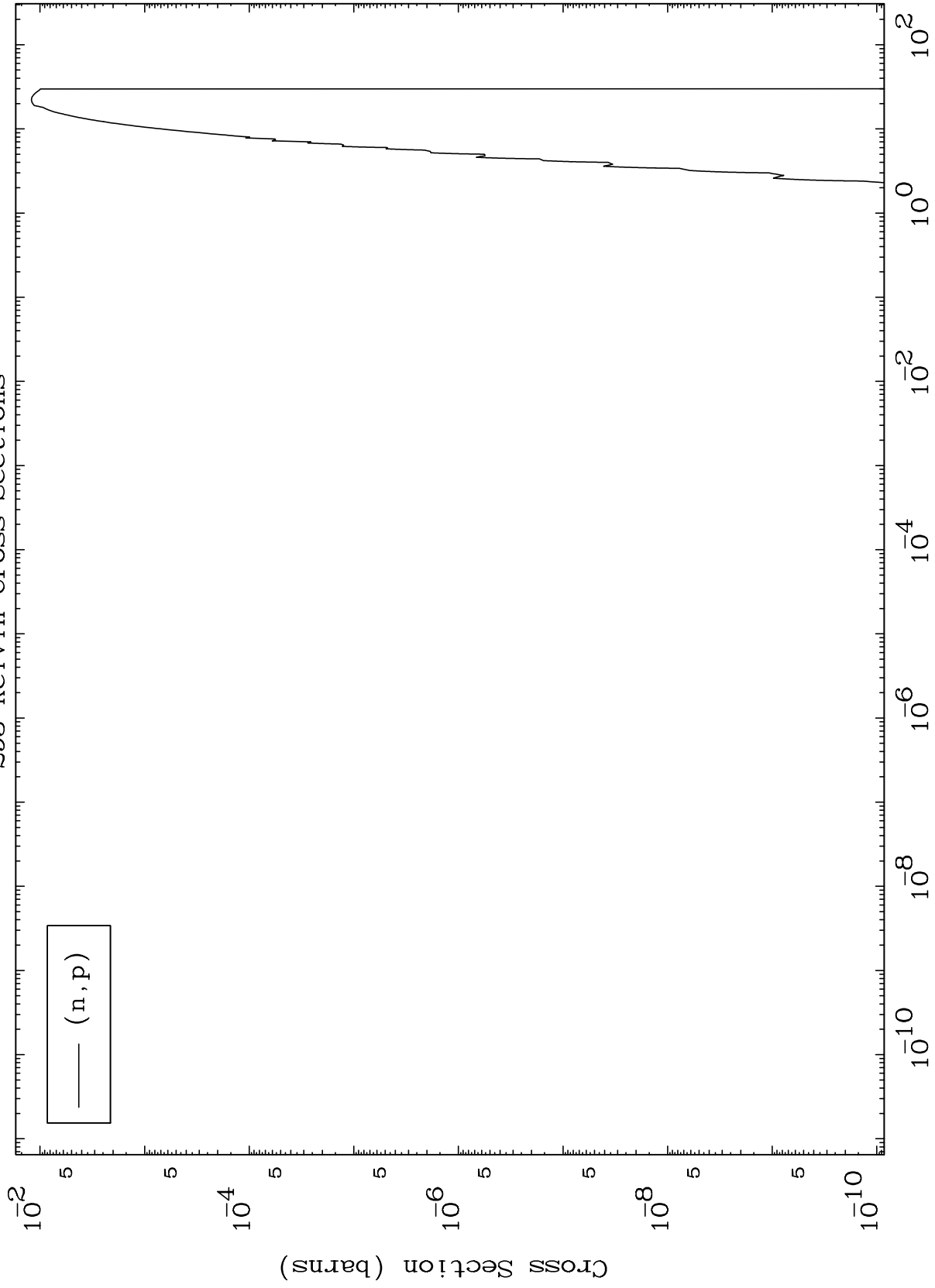
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n,p) Levels
293 Kelvin Cross Sections

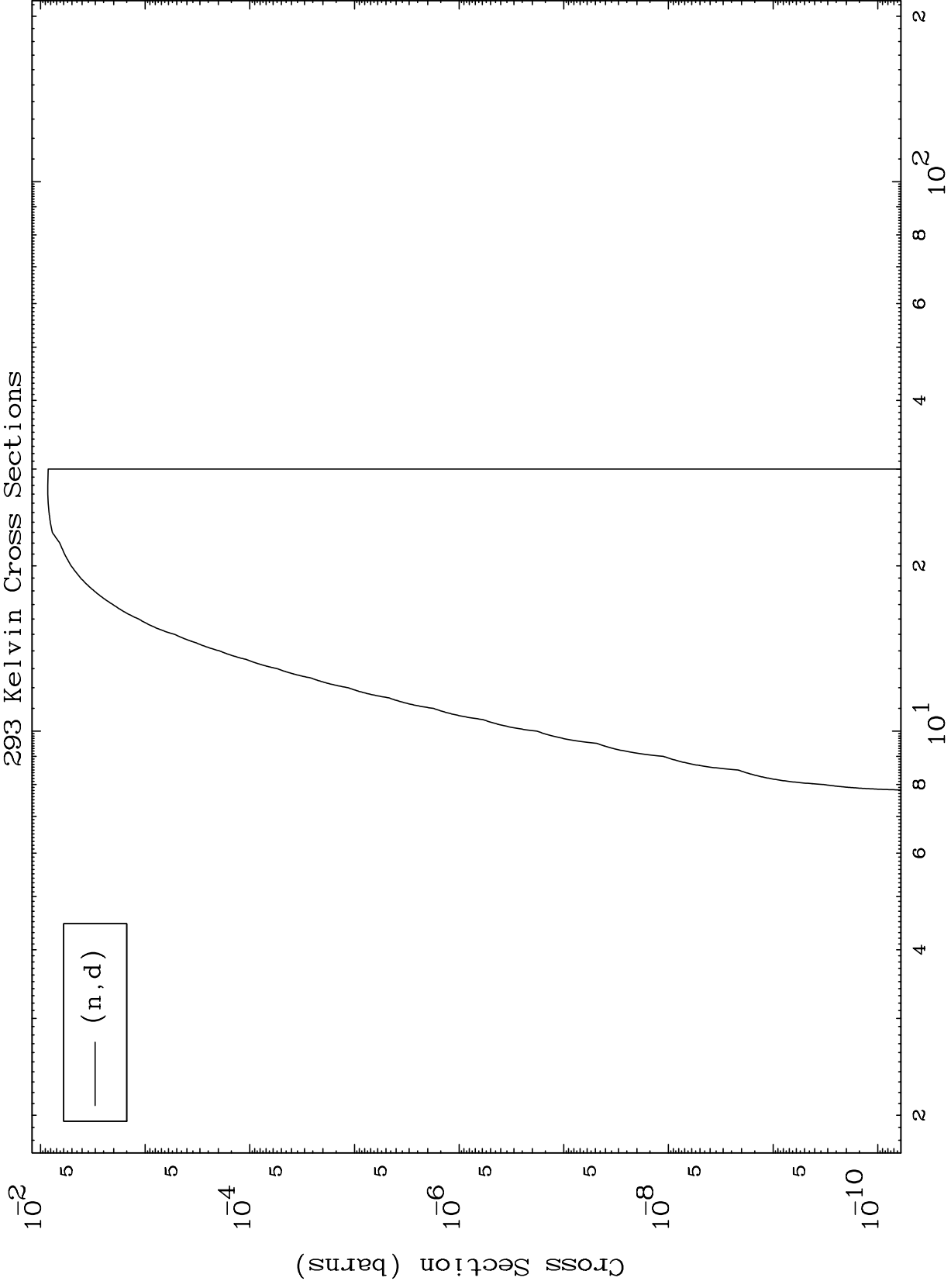
103-Lr-264



MAT 9993

(n,d) Levels
293 Kelvin Cross Sections

103-Lr-264



12

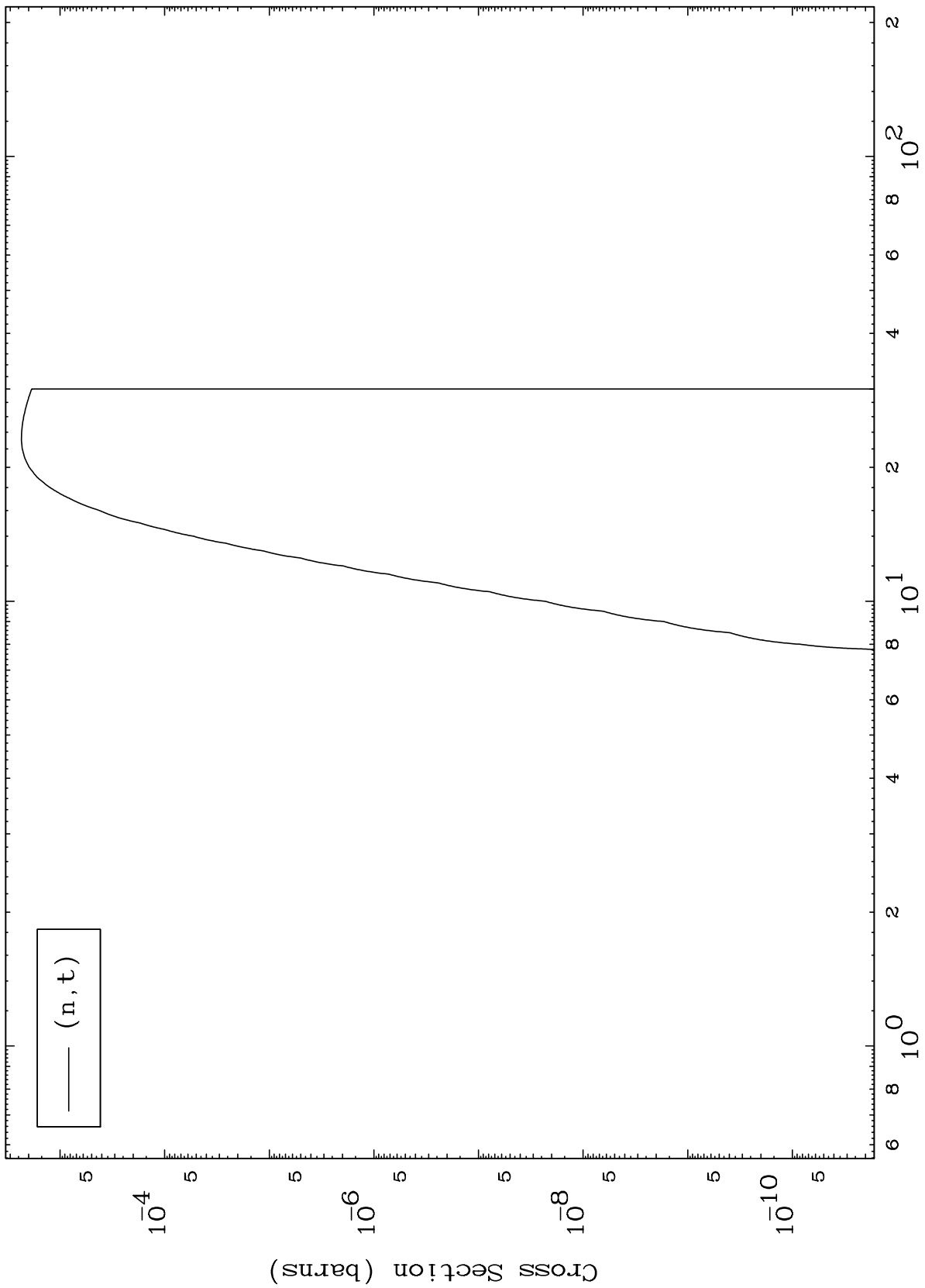
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n,t) Levels
293 Kelvin Cross Sections

103-Lr-264



(n,t)

13

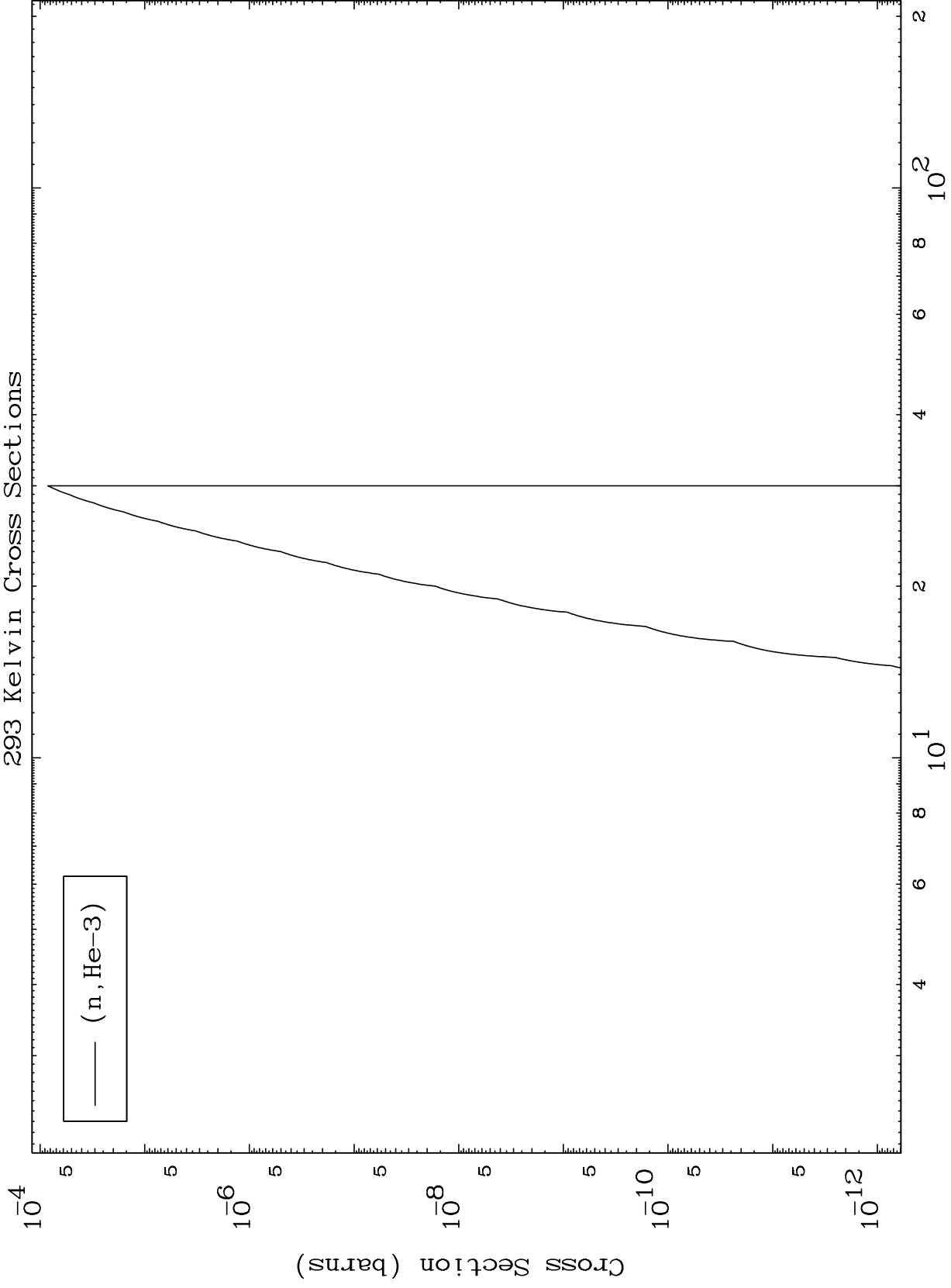
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n,He3) Levels
293 Kelvin Cross Sections

103-Lr-264



14

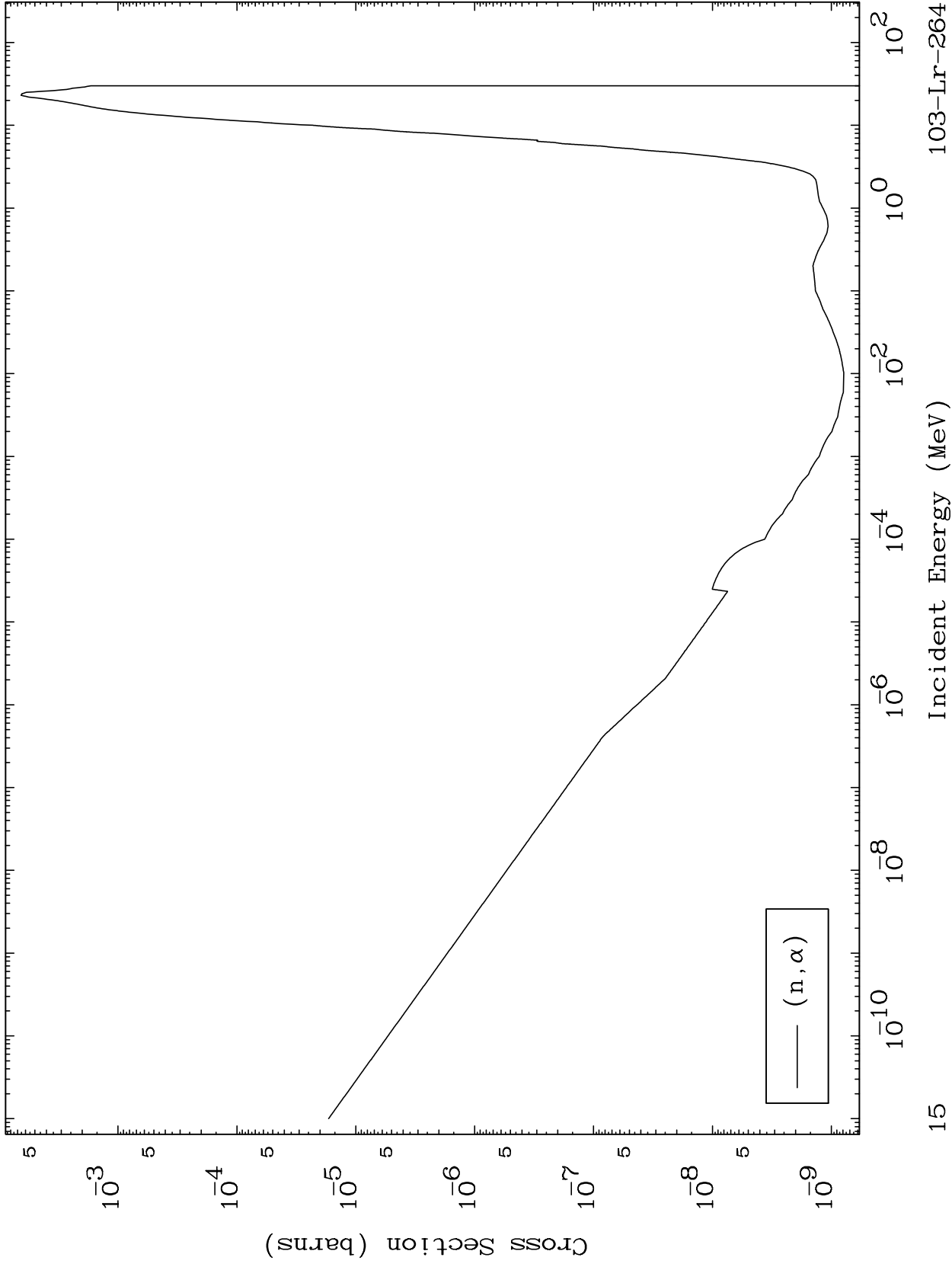
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n, α) Levels
293 Kelvin Cross Sections

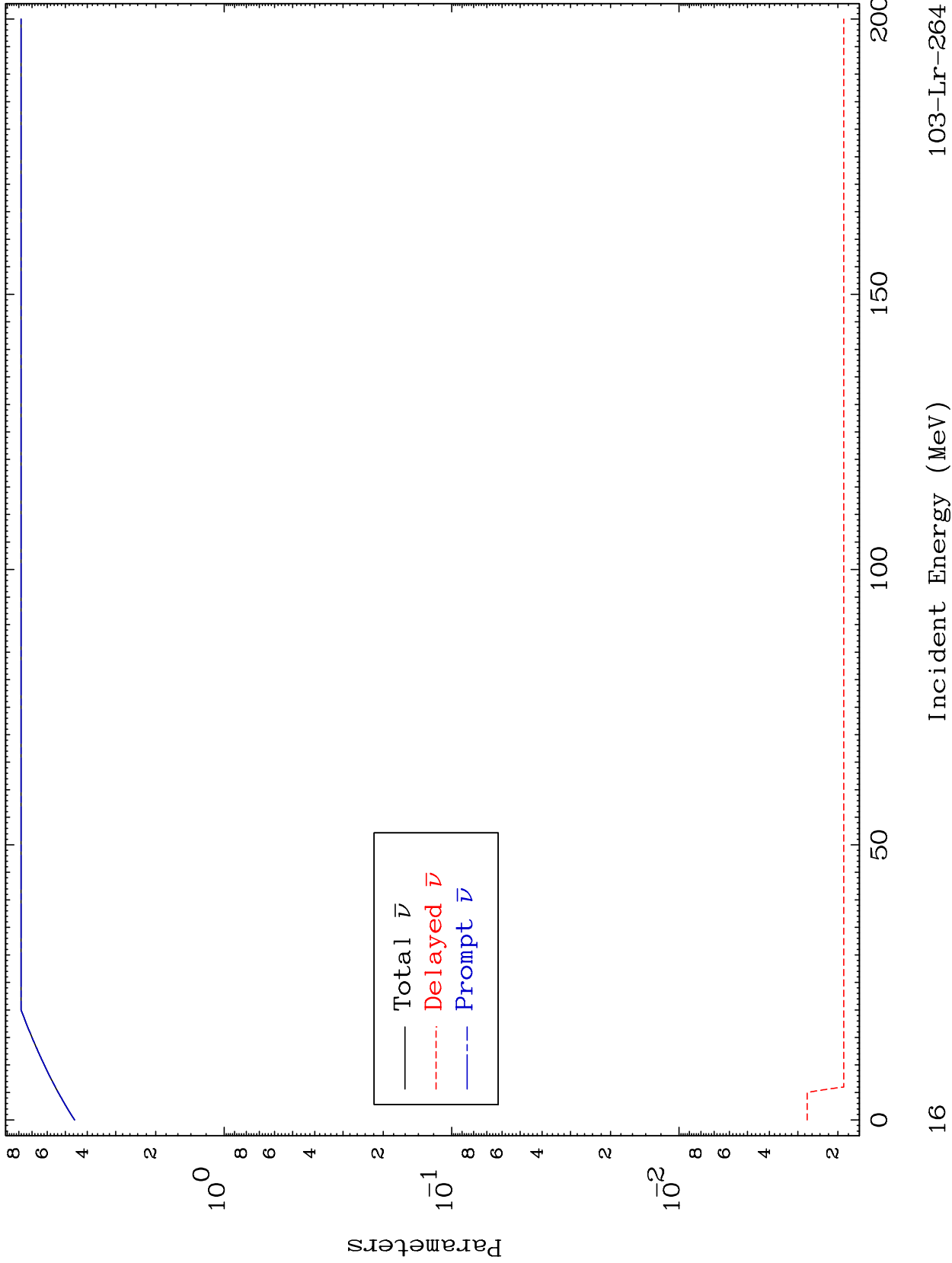
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Energy Release
Parameters

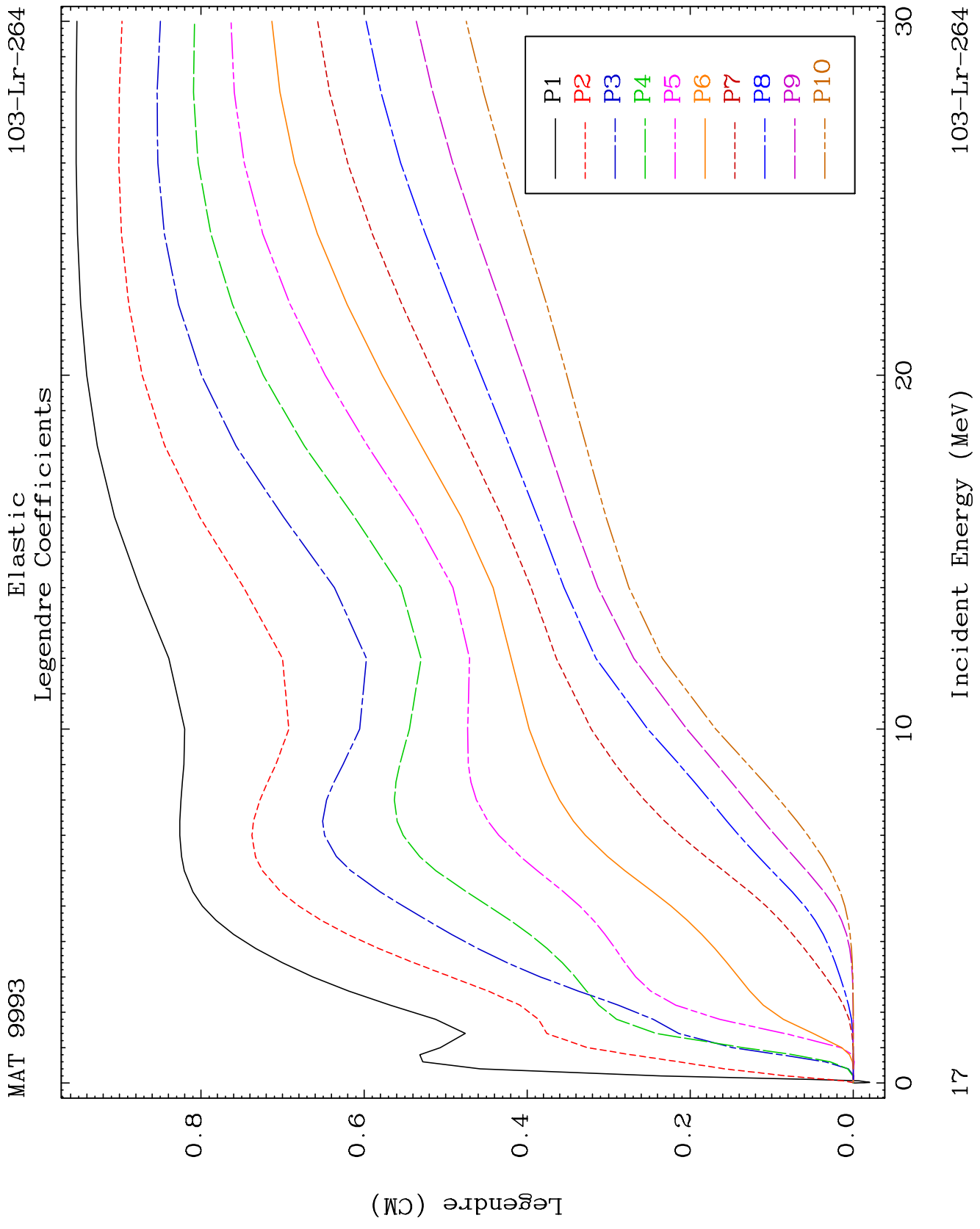
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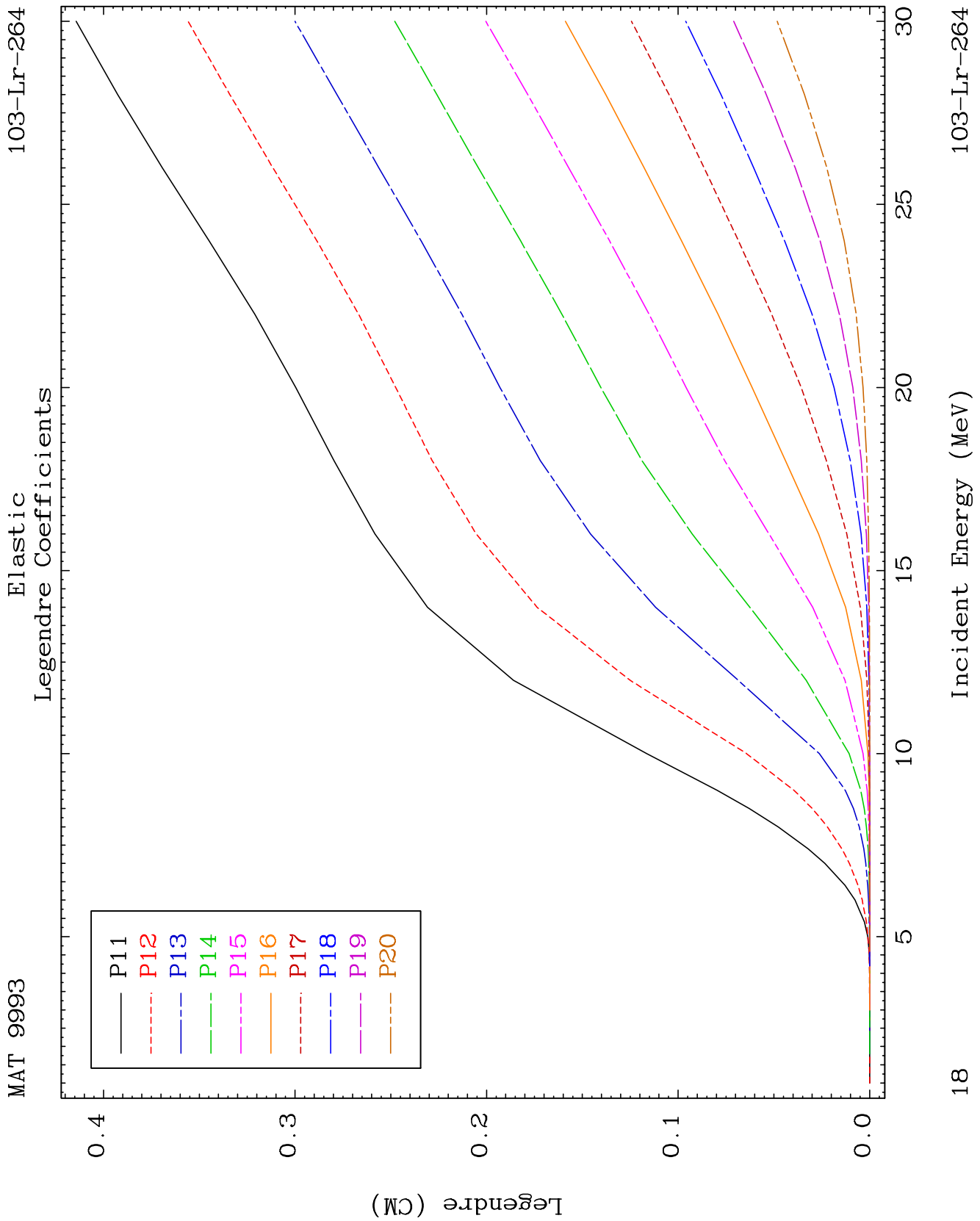


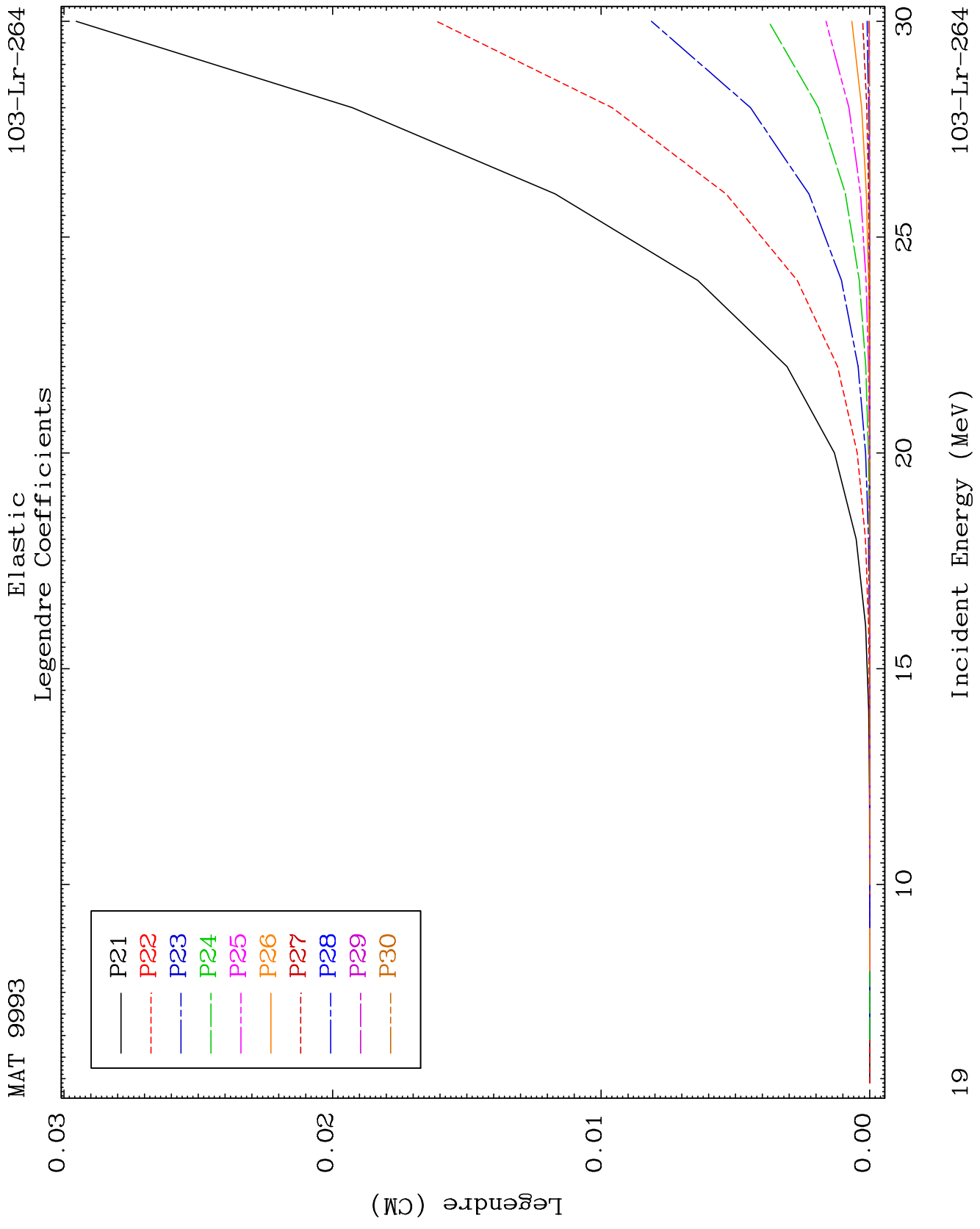
16

Incident Energy (MeV)

103-Lr-264



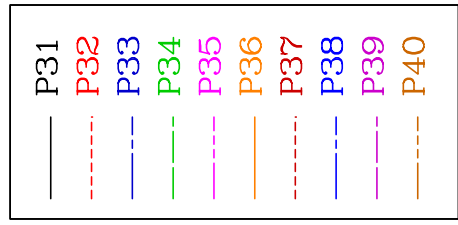




MAT 9993

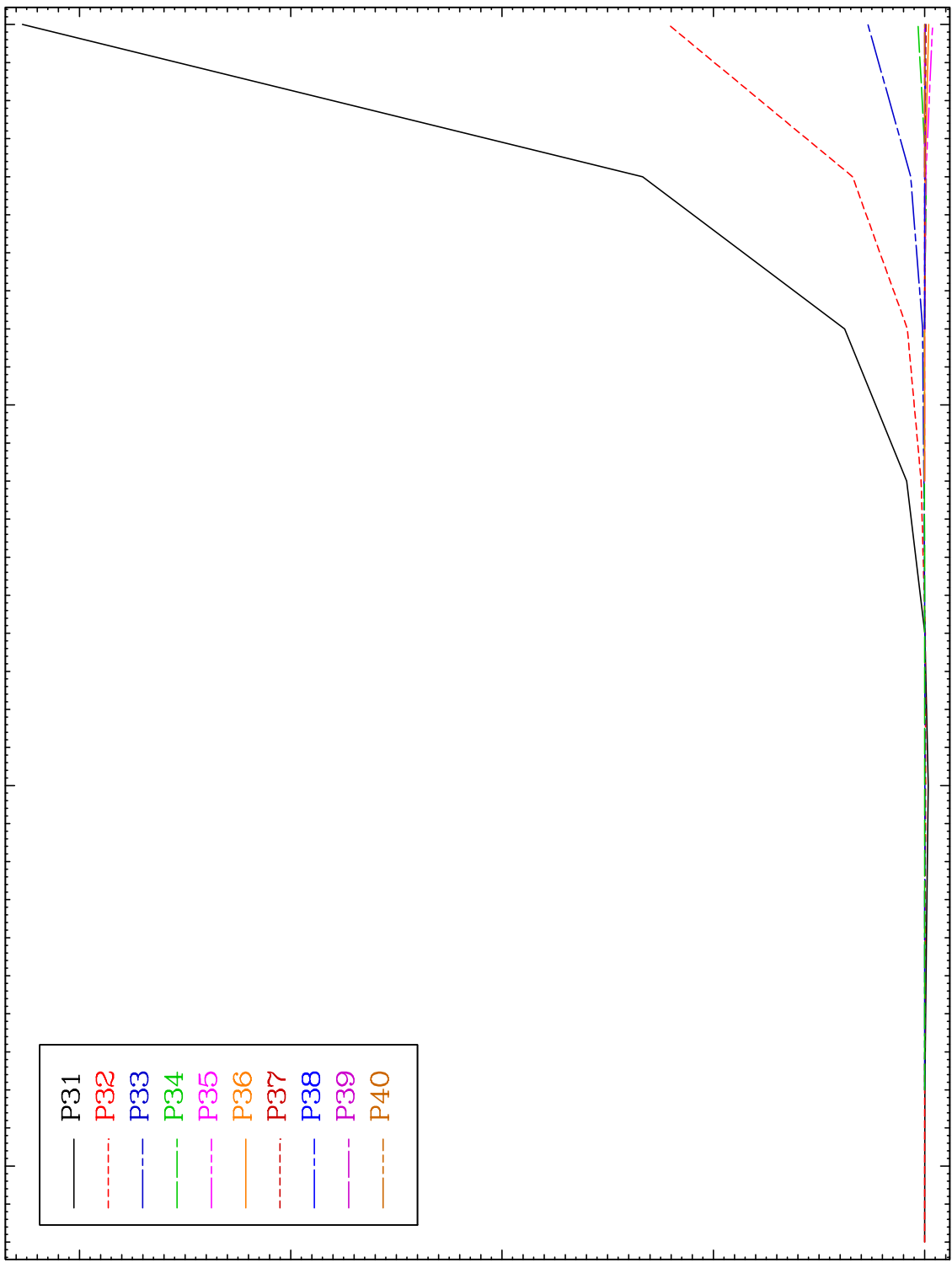
Elastic
Legendre Coefficients

103-Lr-264



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

20

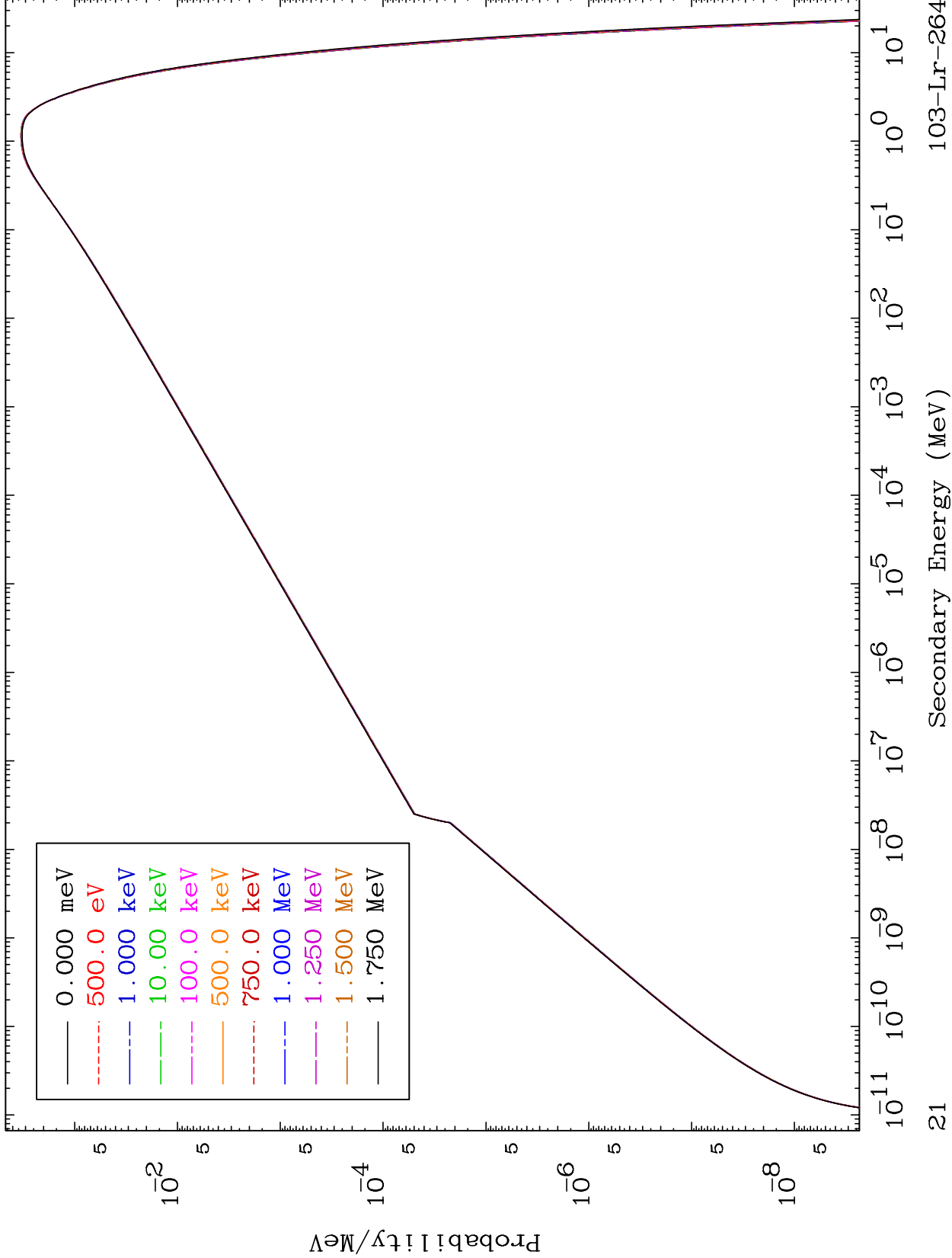
Incident Energy (MeV)

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Fission Energy Distributions

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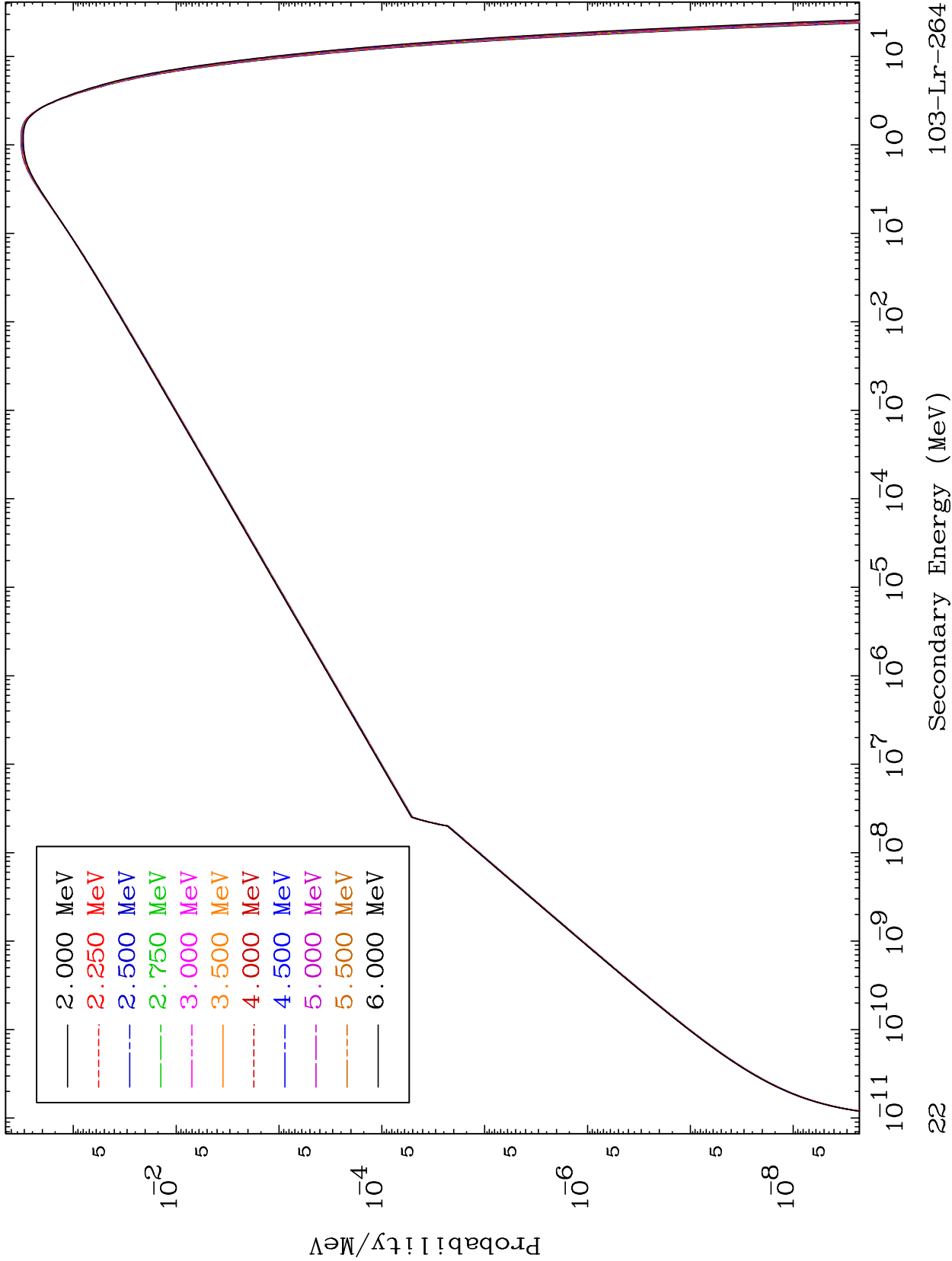


103-Lr-264

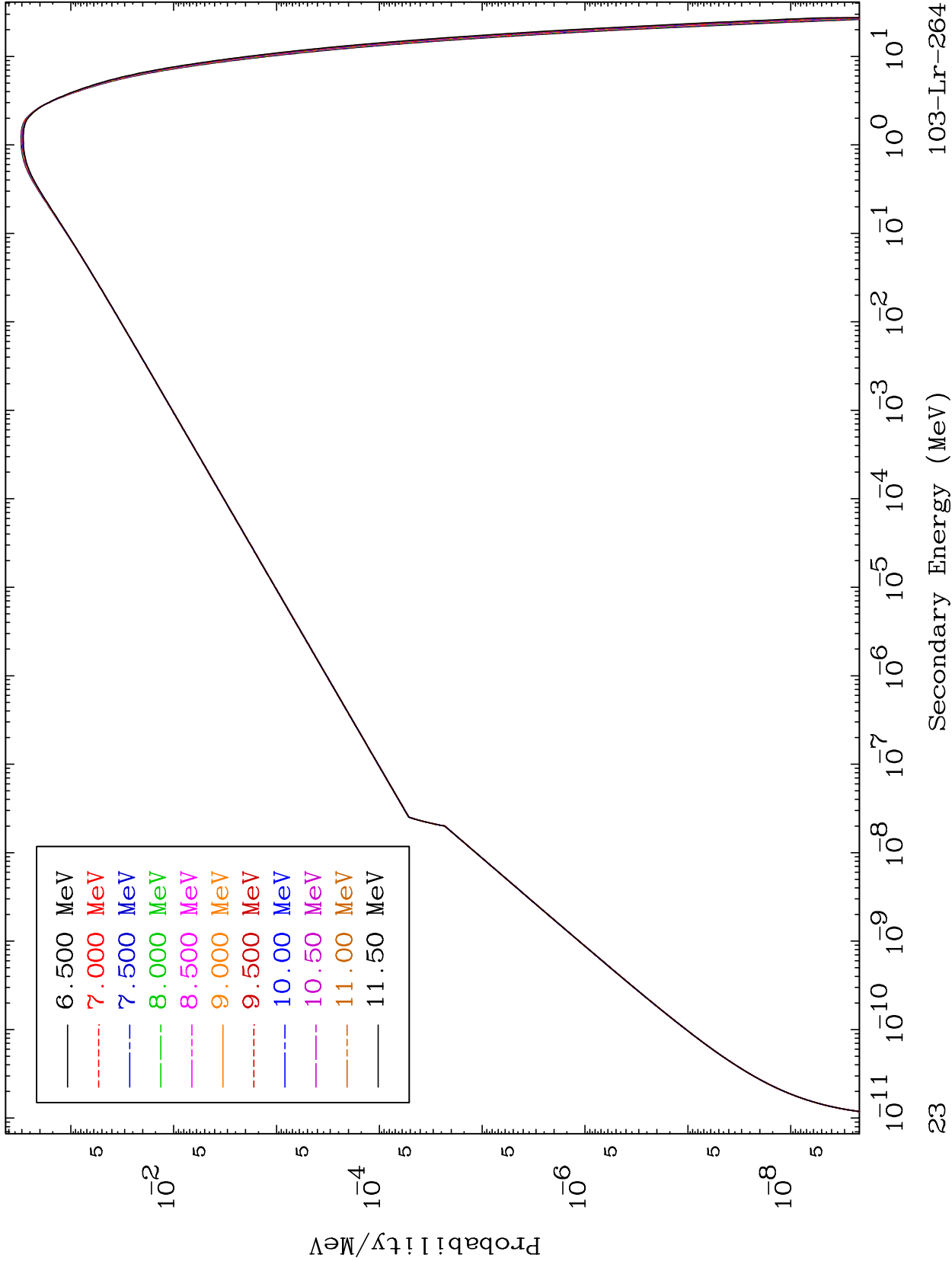
MAT 9993

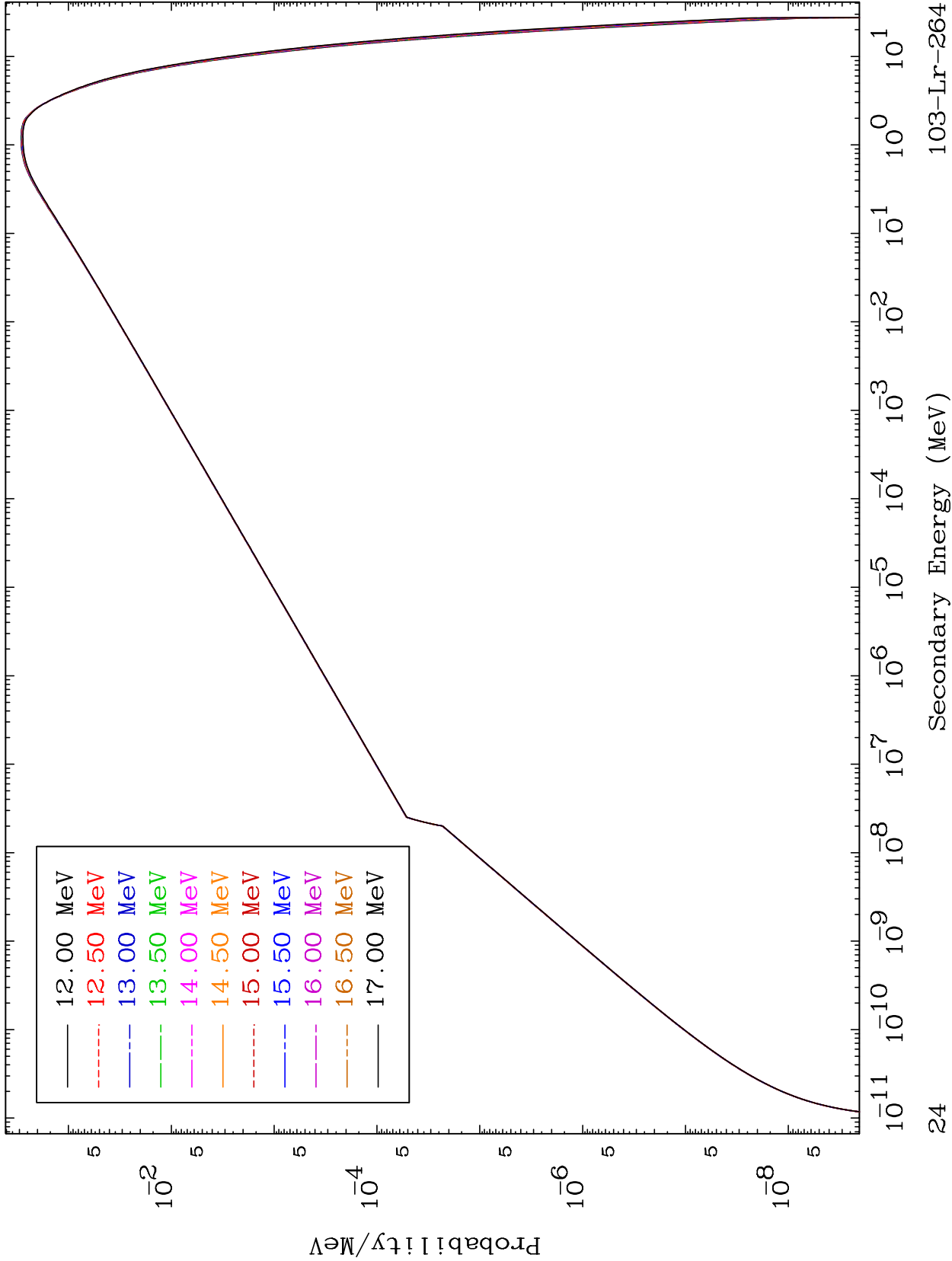
Fission Energy Distributions

103-Lr-264



Fission Energy Distributions

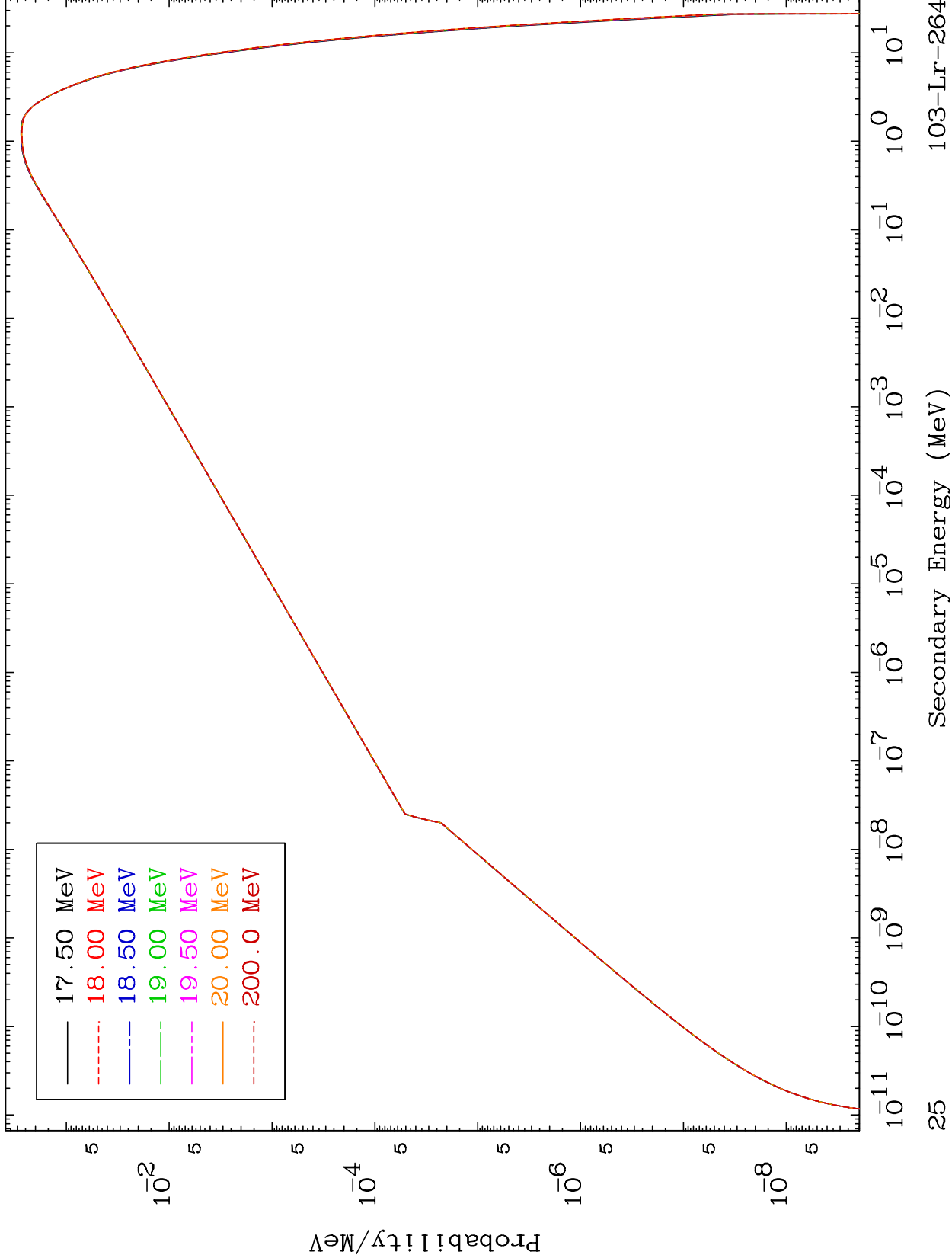




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Fission Energy Distributions

103-Lr-264

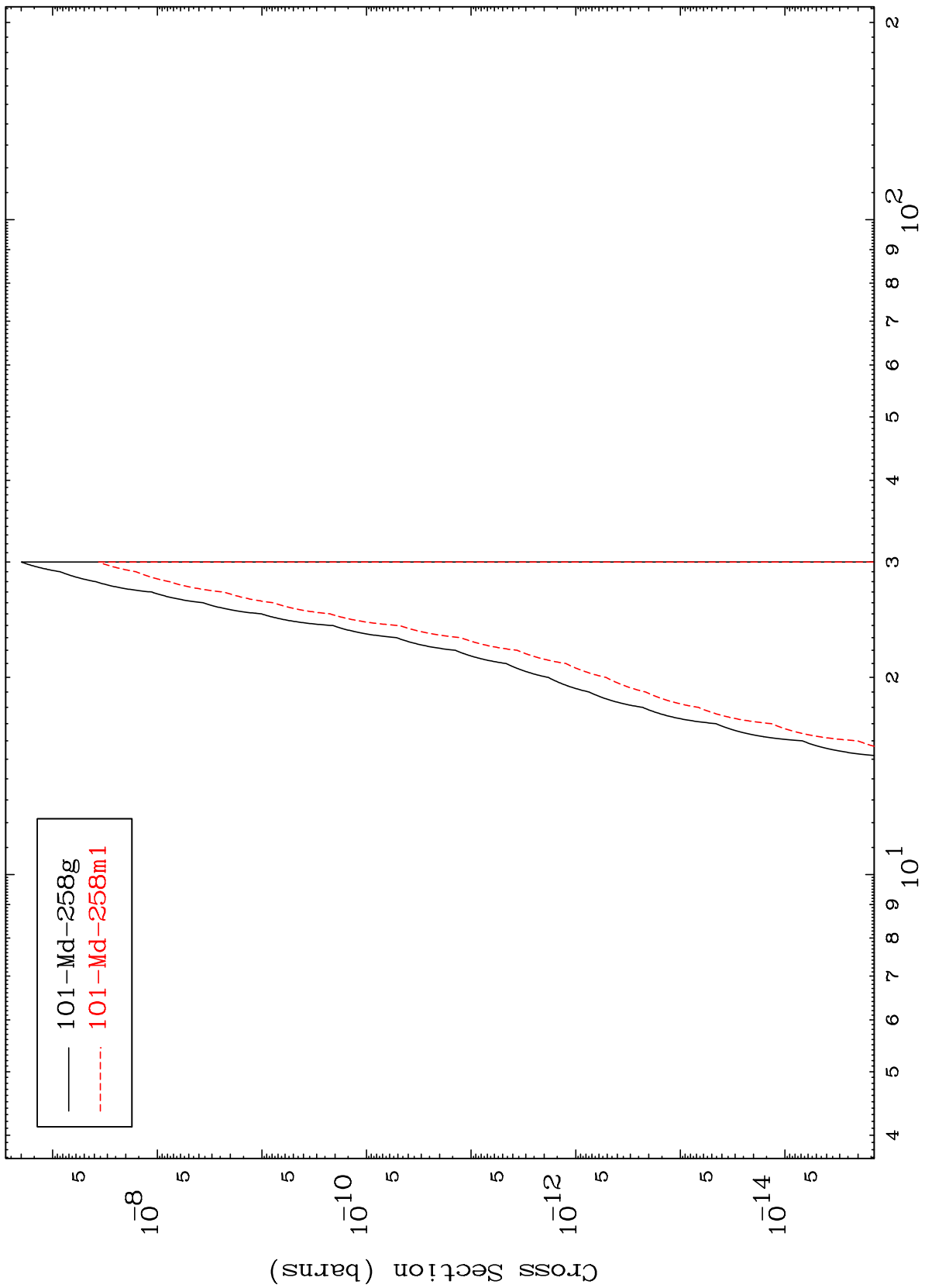


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$(n,3n) \alpha$

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

103-Lr-264