

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

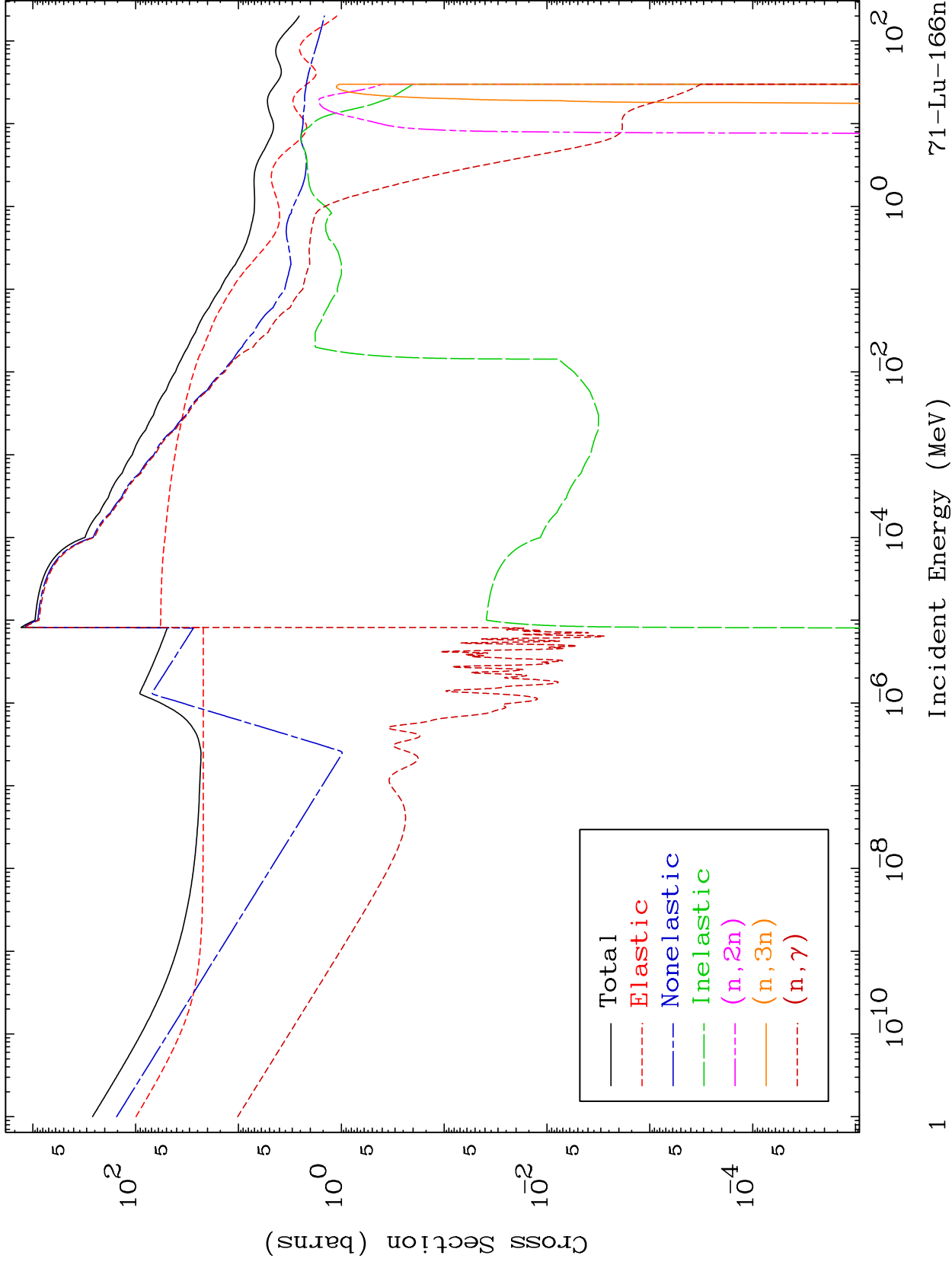
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7100

Neutron Major
293 Kelvin Cross Sections

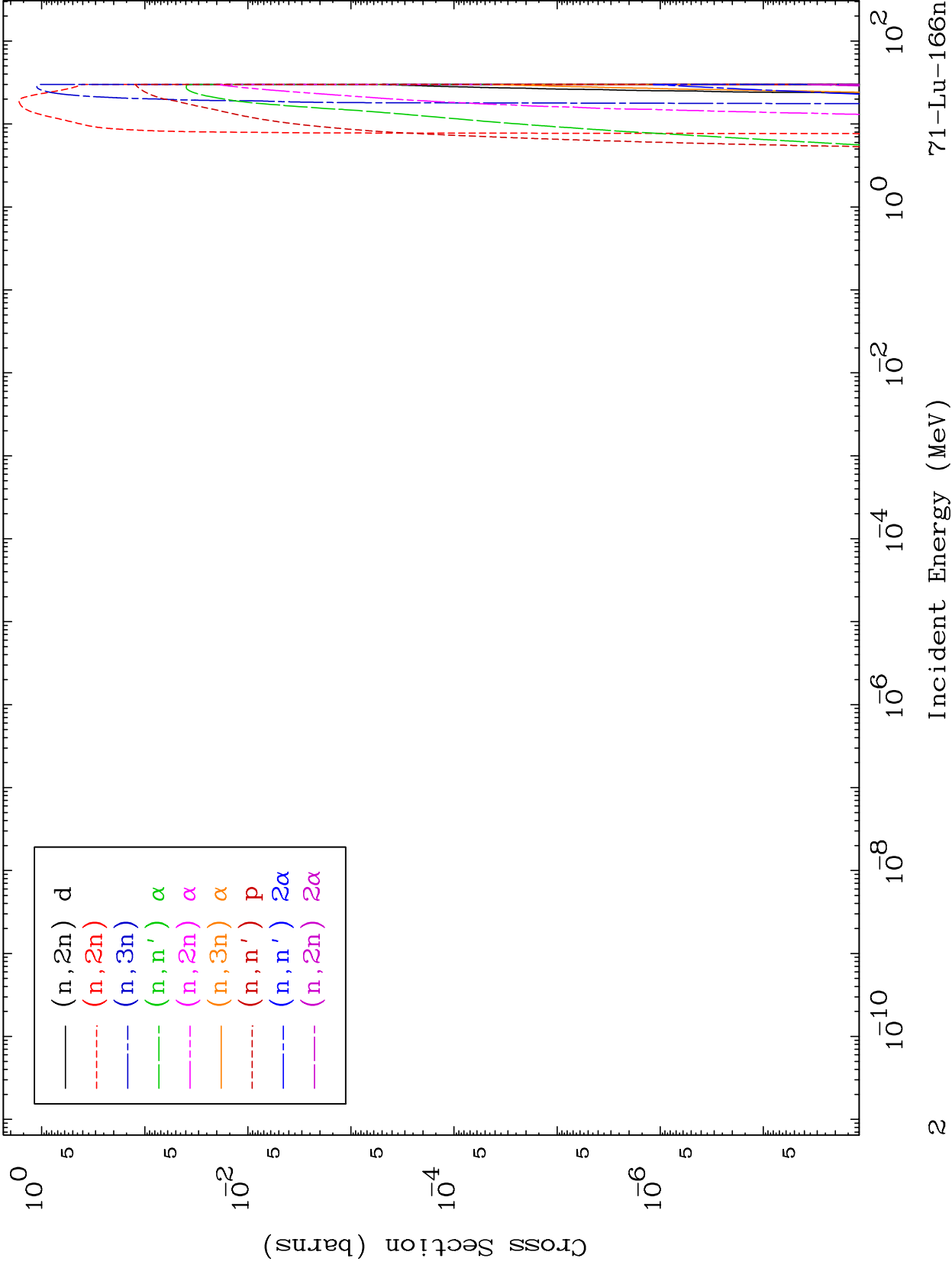
71-Lu-166n



MAT 7100

Neutron Absorption
293 Kelvin Cross Sections

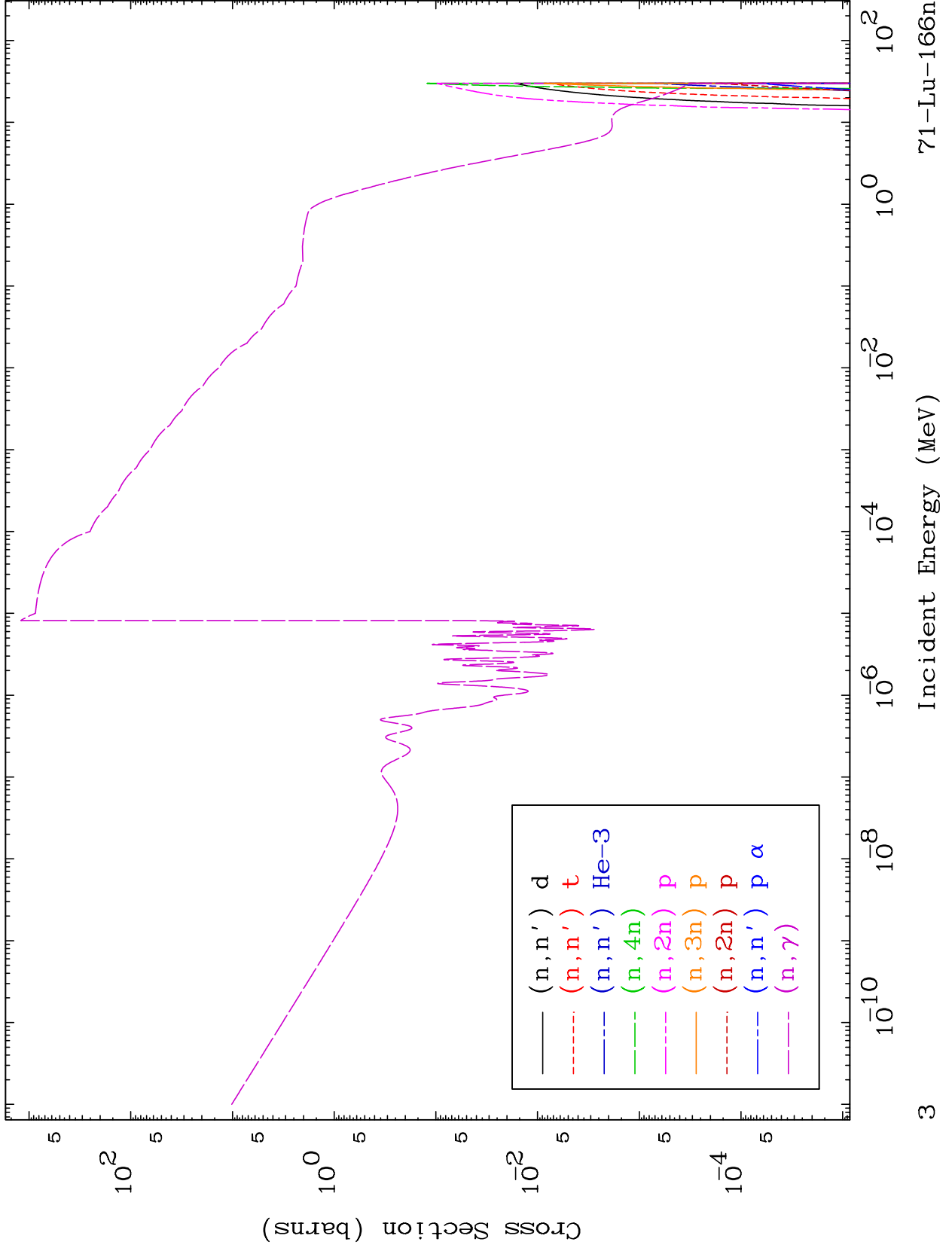
⁷¹Lu-166n

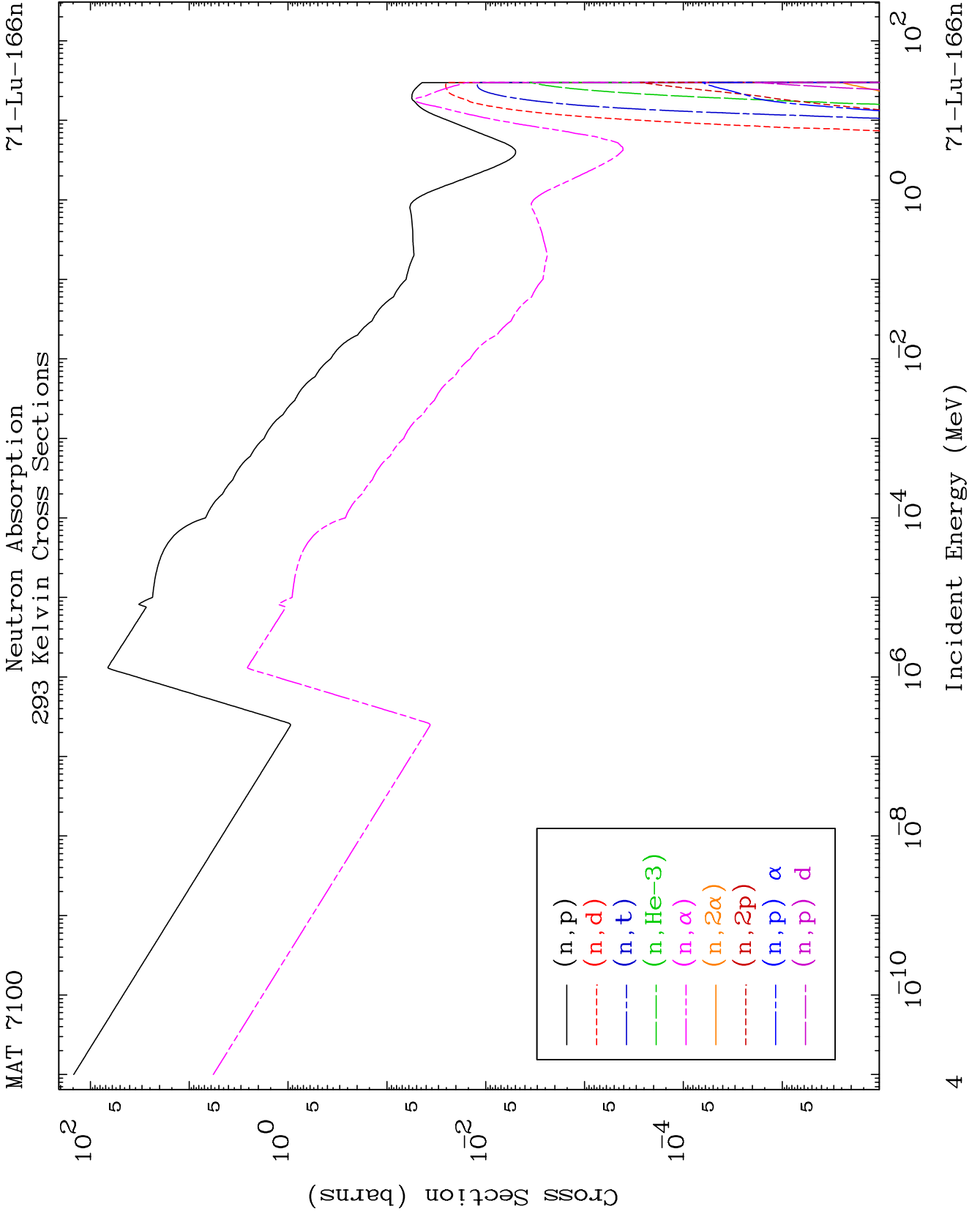


MAT 7100

Neutron Absorption
293 Kelvin Cross Sections

71-Lu-166n

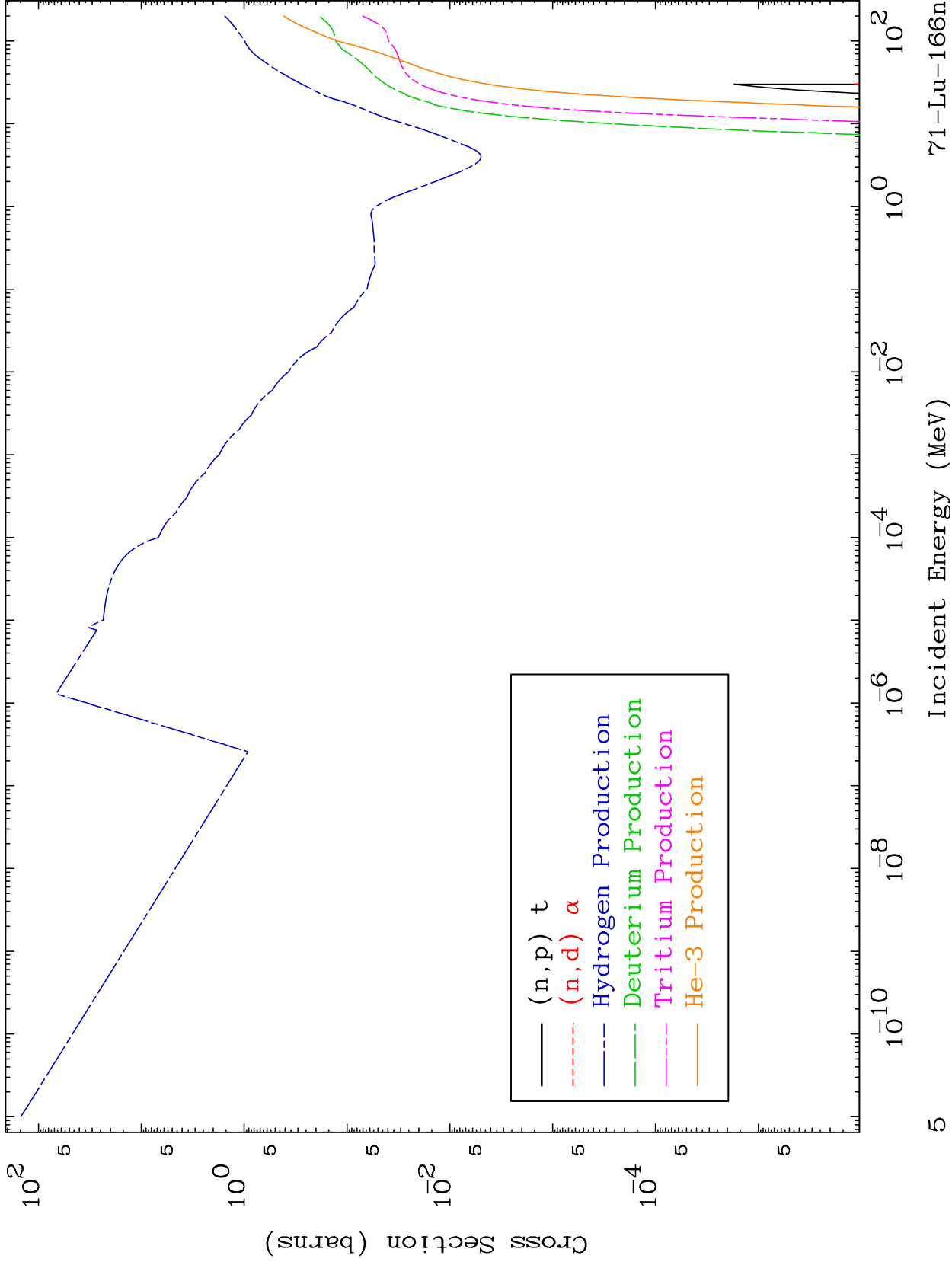




MAT 7100

Neutron Absorption
293 Kelvin Cross Sections

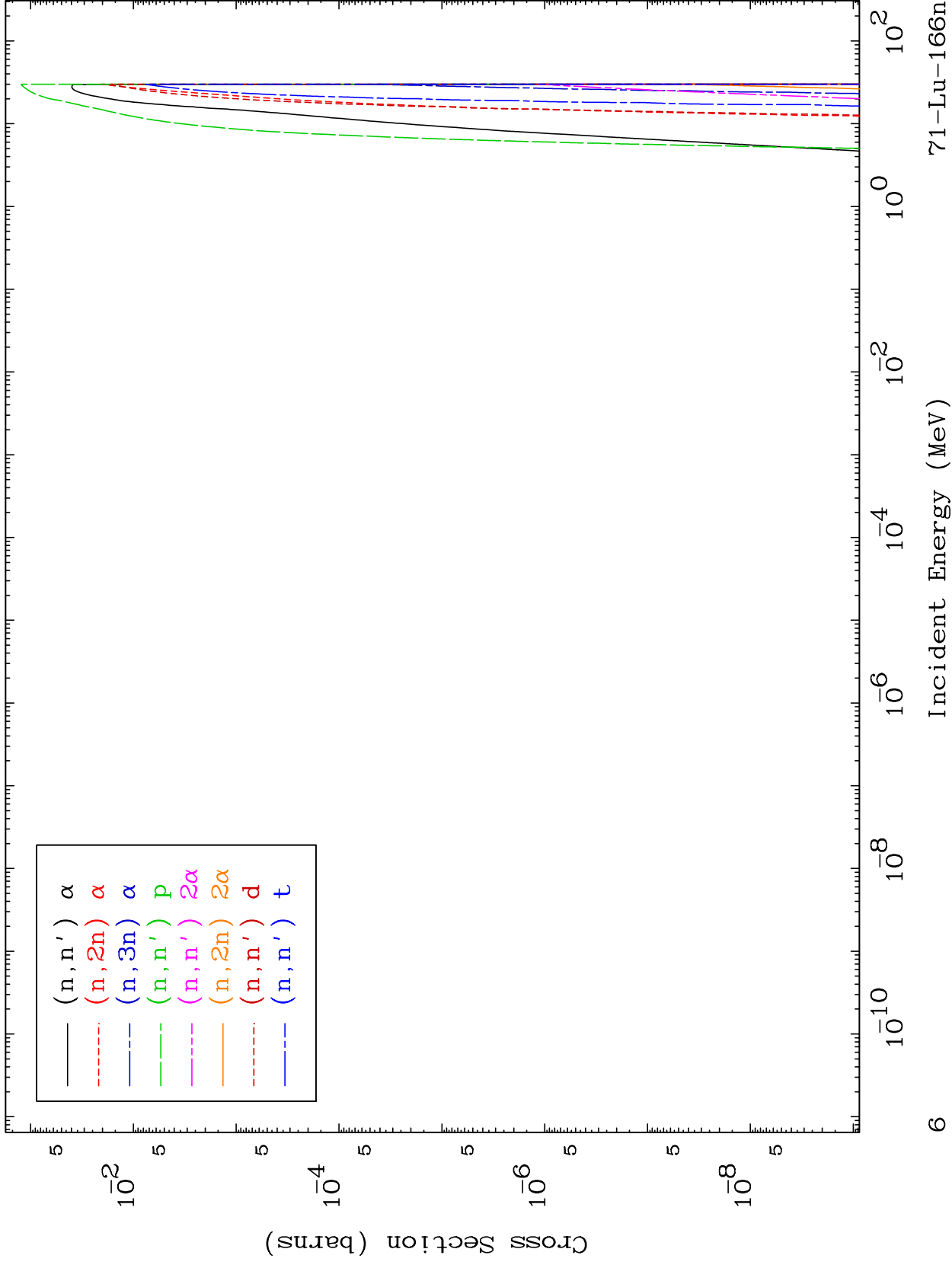
⁷¹Lu-166n

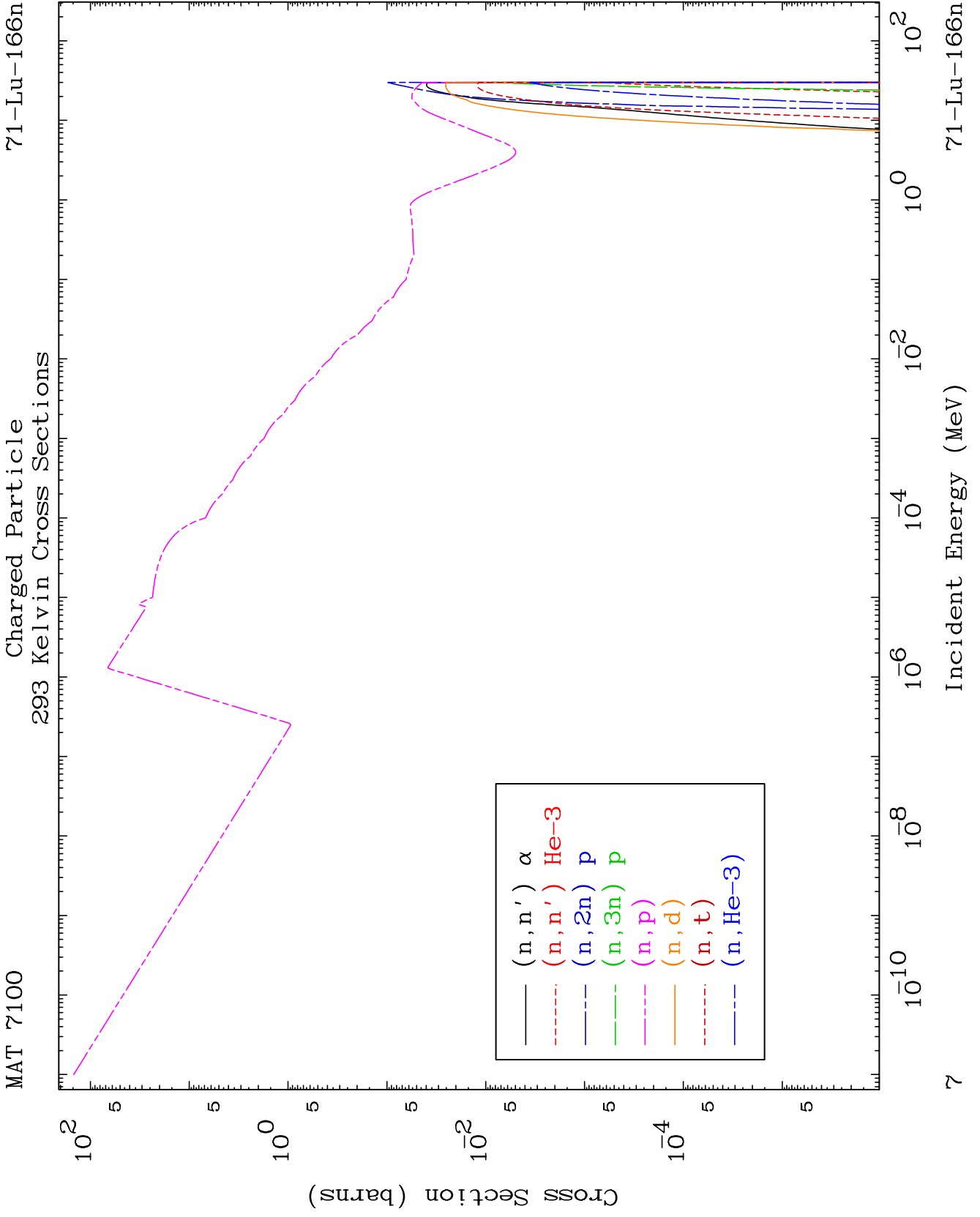


MAT 7100

Charged Particle
293 Kelvin Cross Sections

⁷¹Lu-166n

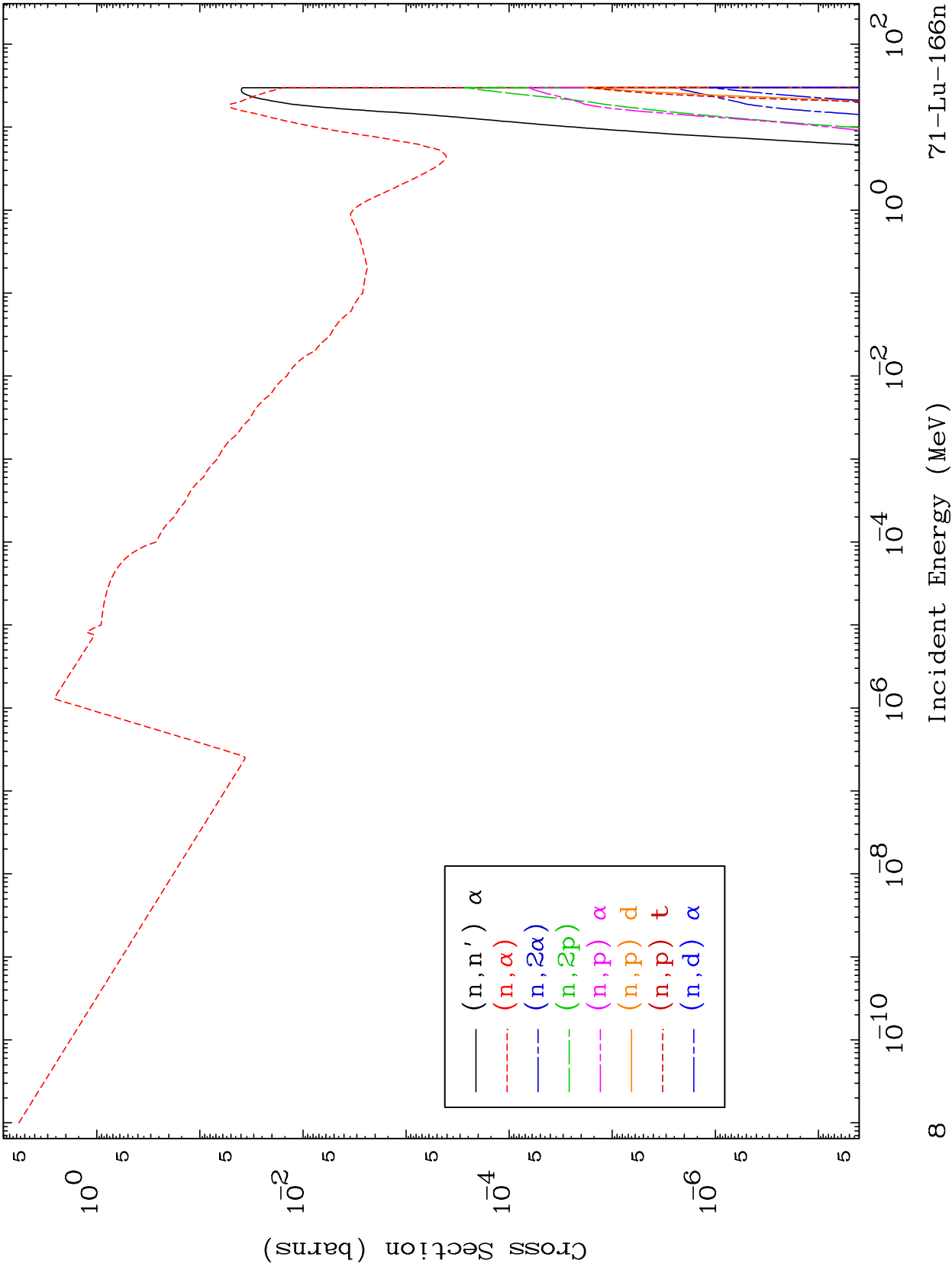




MAT 7100

Charged Particle
293 Kelvin Cross Sections

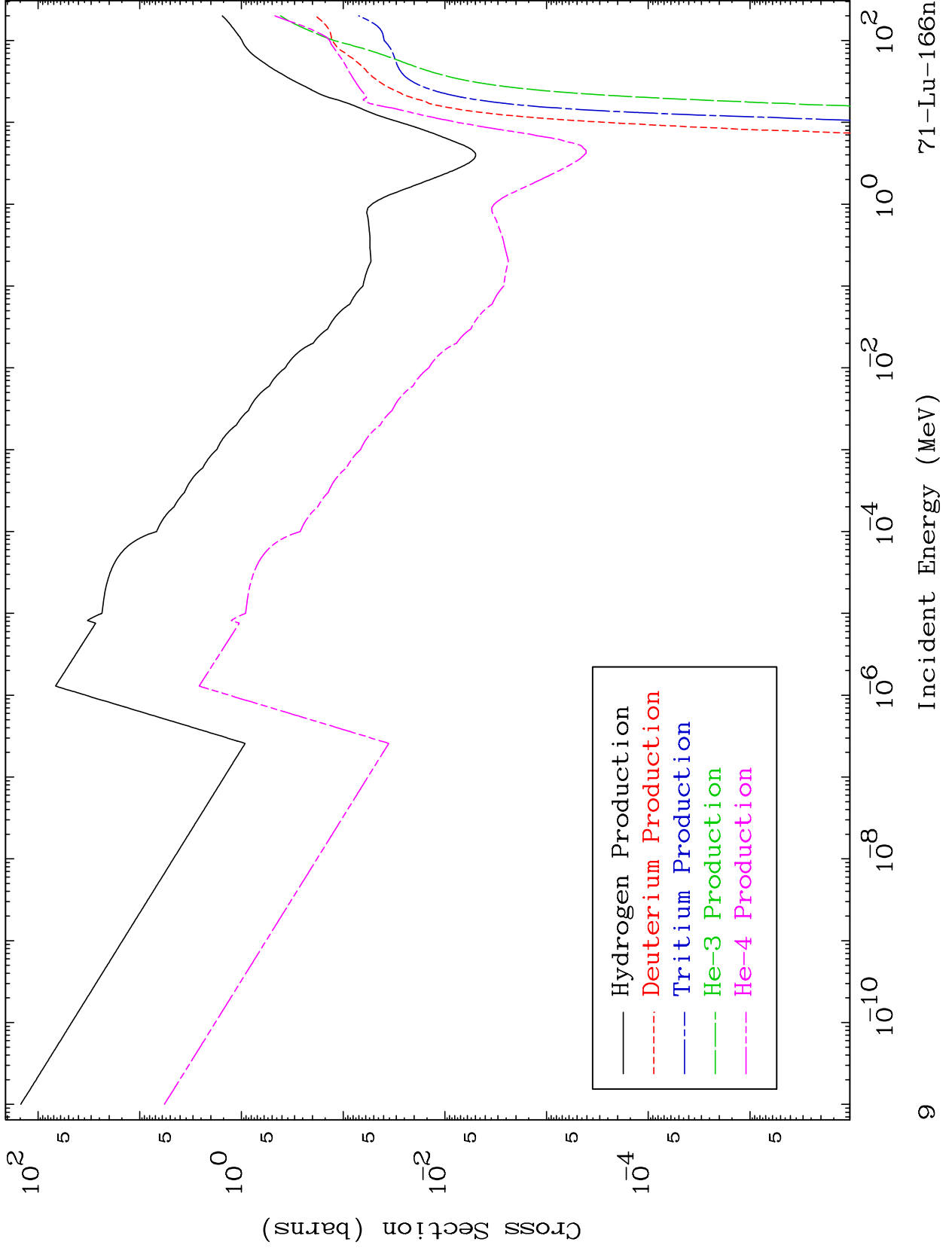
⁷¹Lu-166n



MAT 7100

Particle Production
293 Kelvin Cross Sections

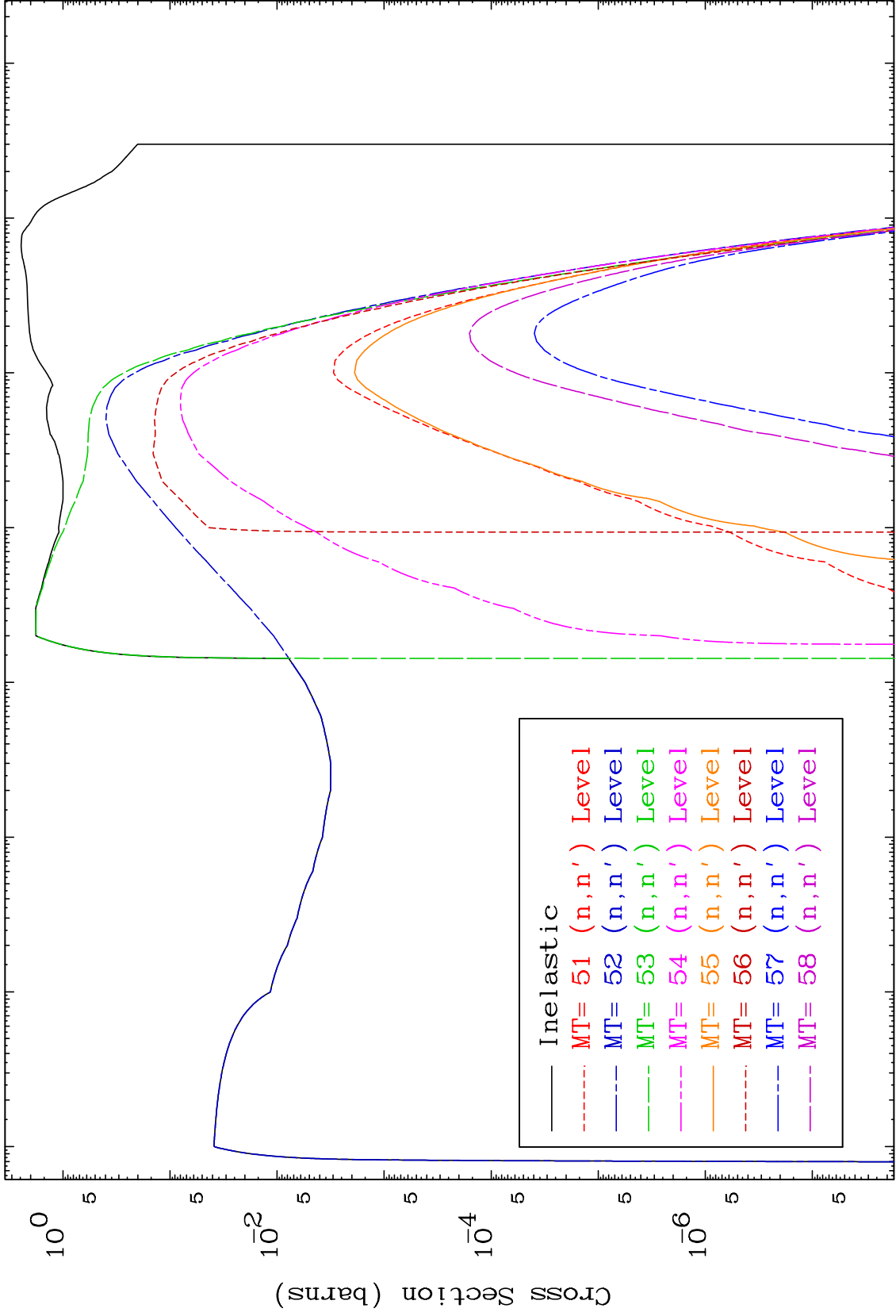
⁷¹Lu-166n



MAT 7100

293 Kelvin Cross Sections
(n,n') Levels

71-Lu-166n



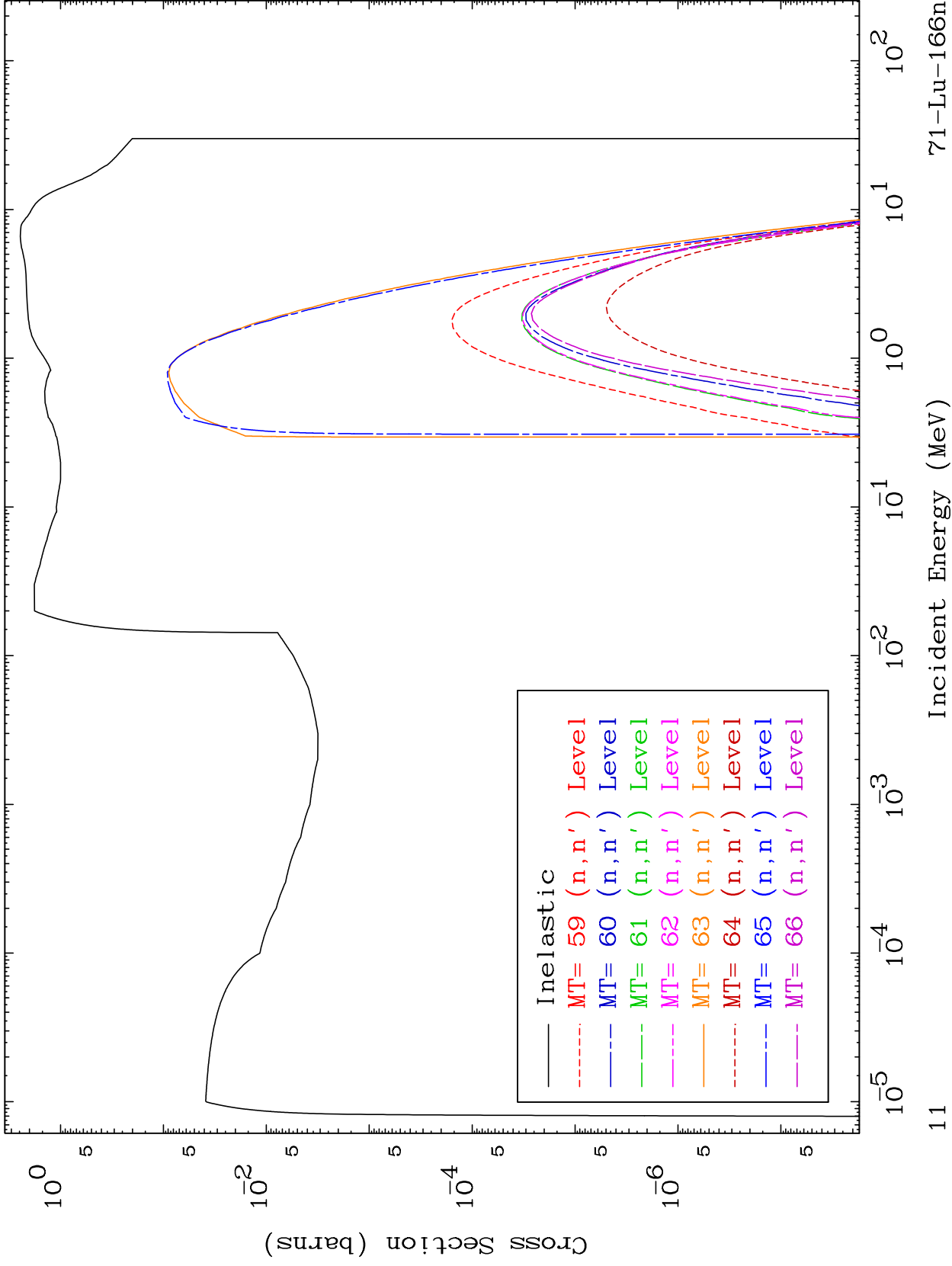
71-Lu-166n

Incident Energy (MeV)

MAT 7100

(n,n') Levels
293 Kelvin Cross Sections

71-Lu-166n

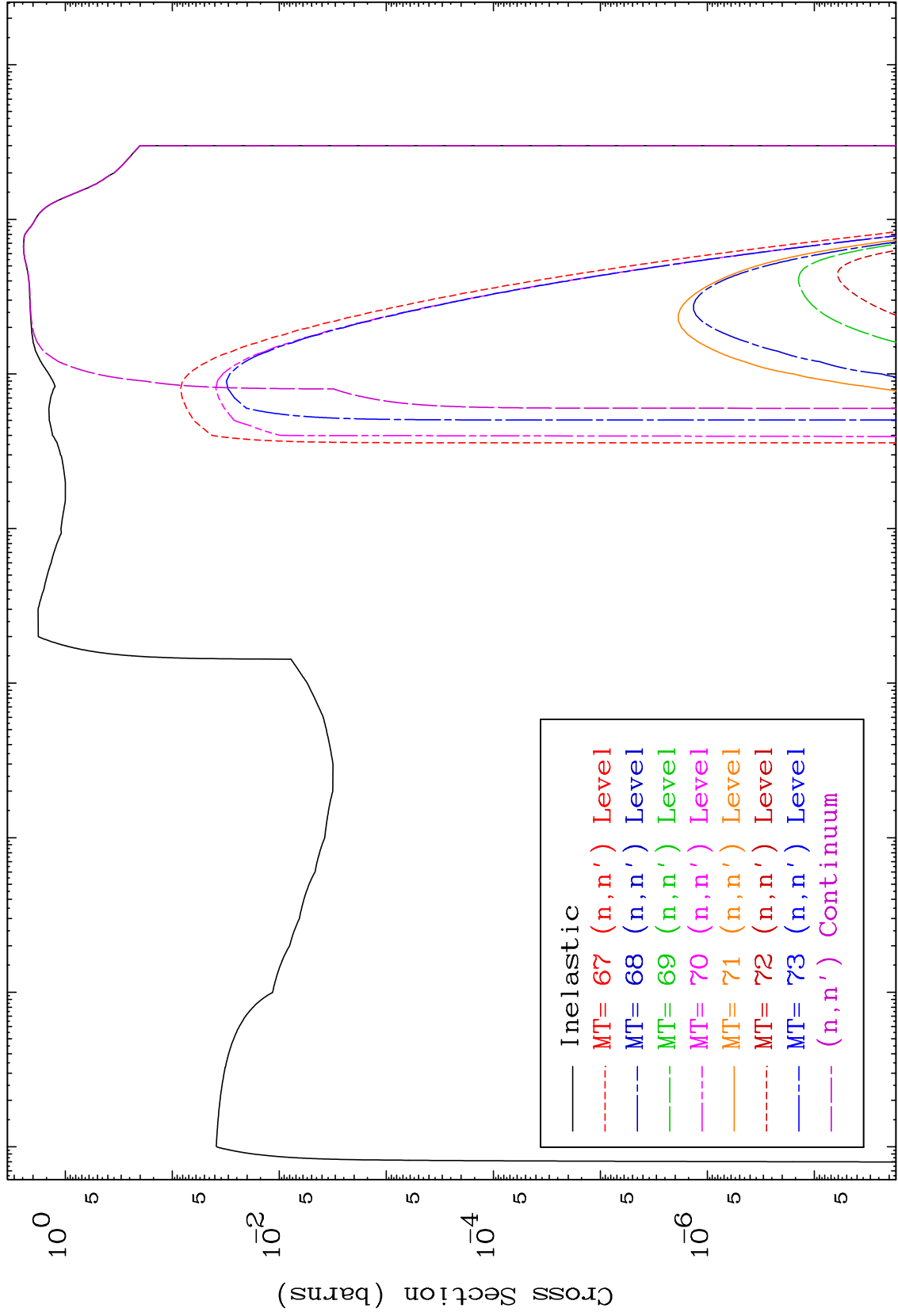


MAT 7100

(n,n') Levels

71-Lu-166n

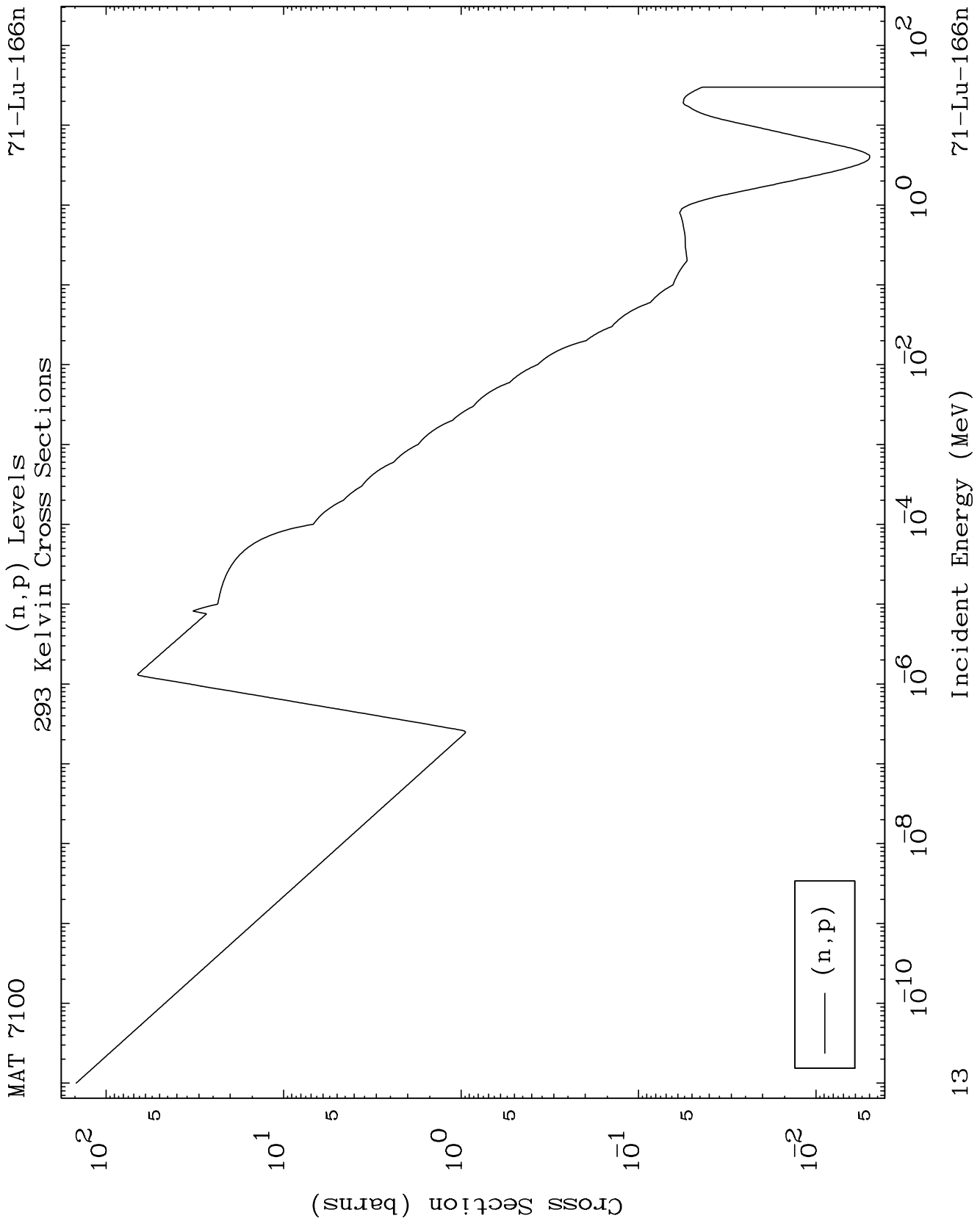
293 Kelvin Cross Sections



71-Lu-166n

Incident Energy (MeV)

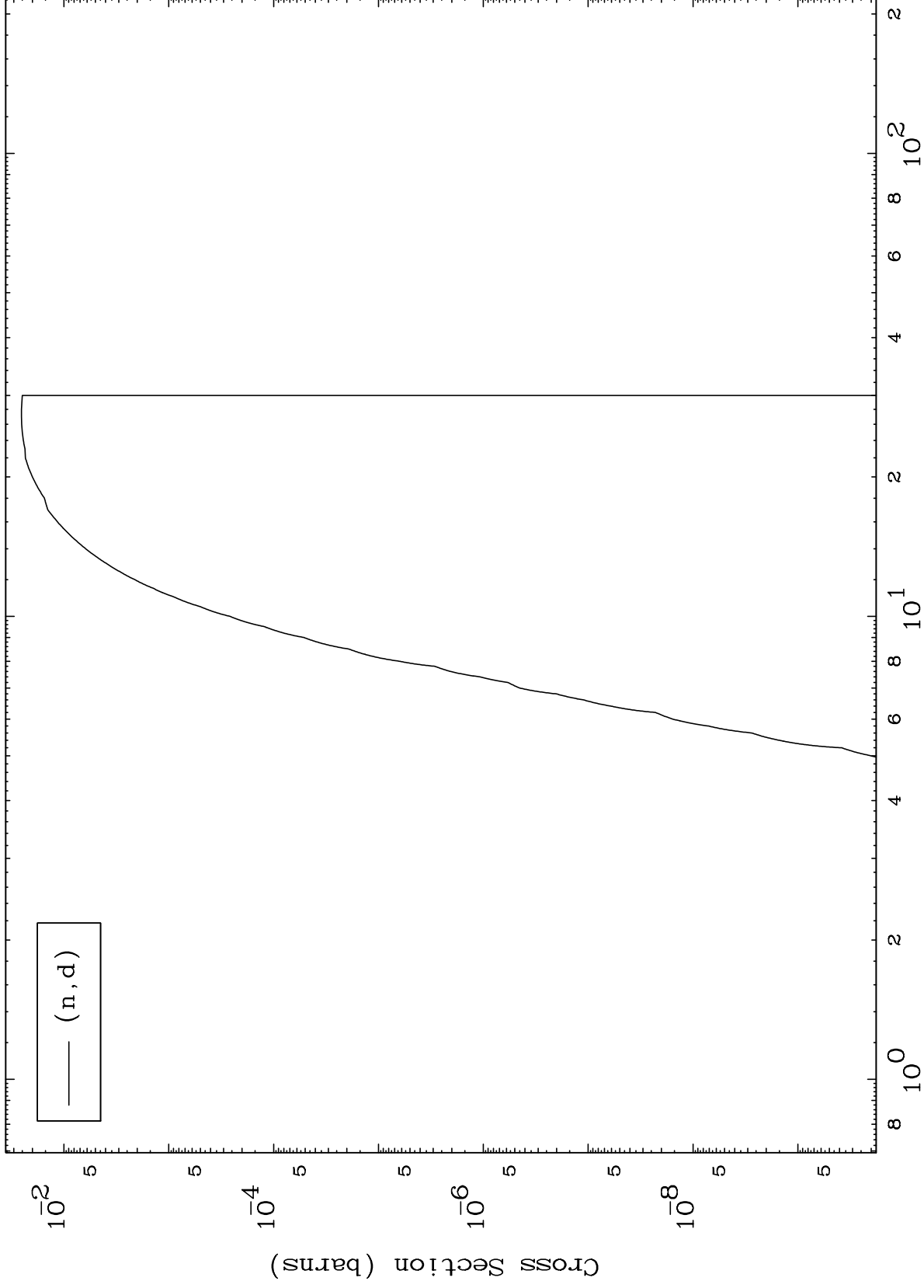
12



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(n,d) Levels
293 Kelvin Cross Sections

71-Lu-166n



14

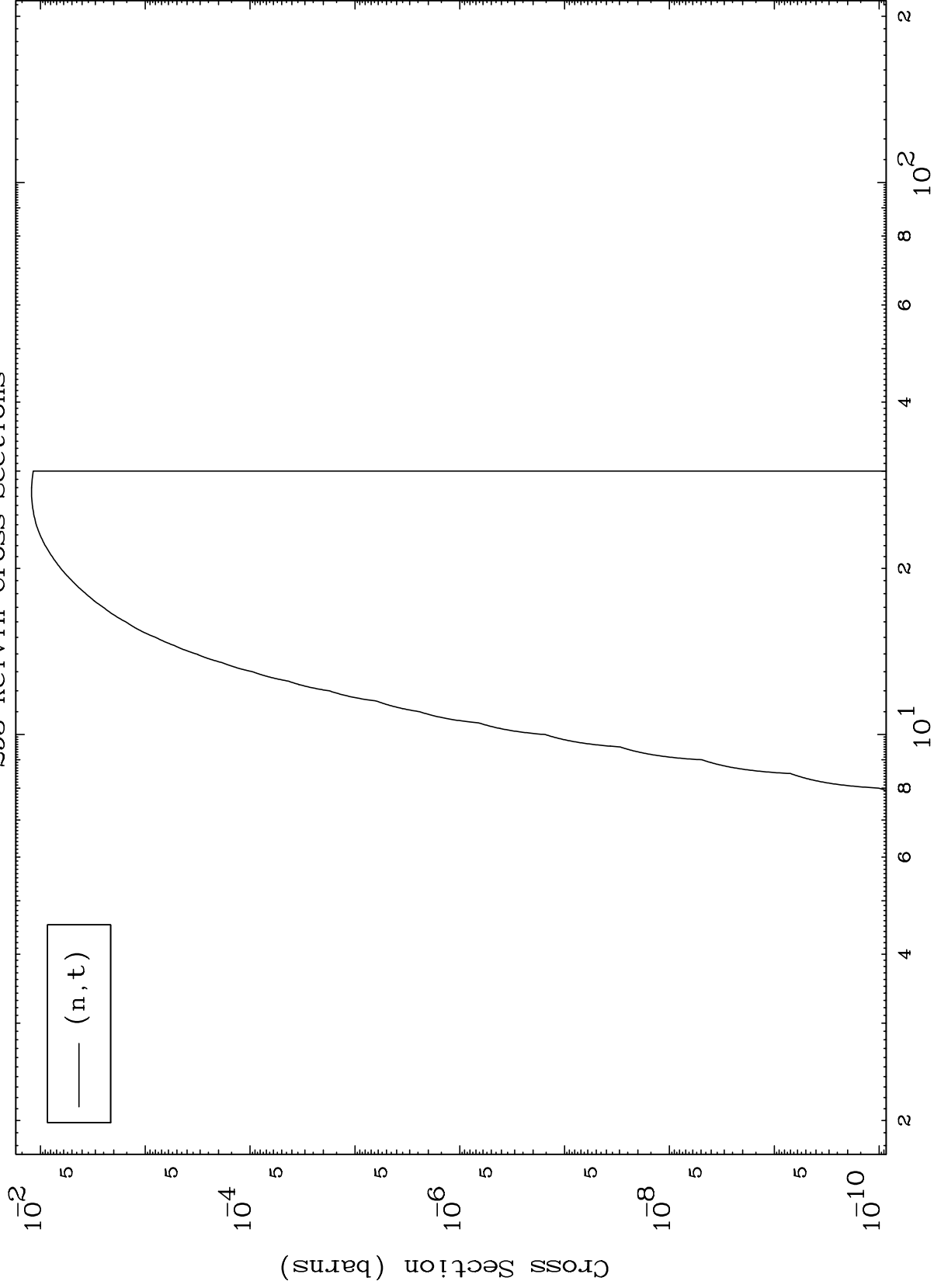
Incident Energy (MeV)

71-Lu-166n

MAT 7100

(n,t) Levels
293 Kelvin Cross Sections

71-Lu-166n



15

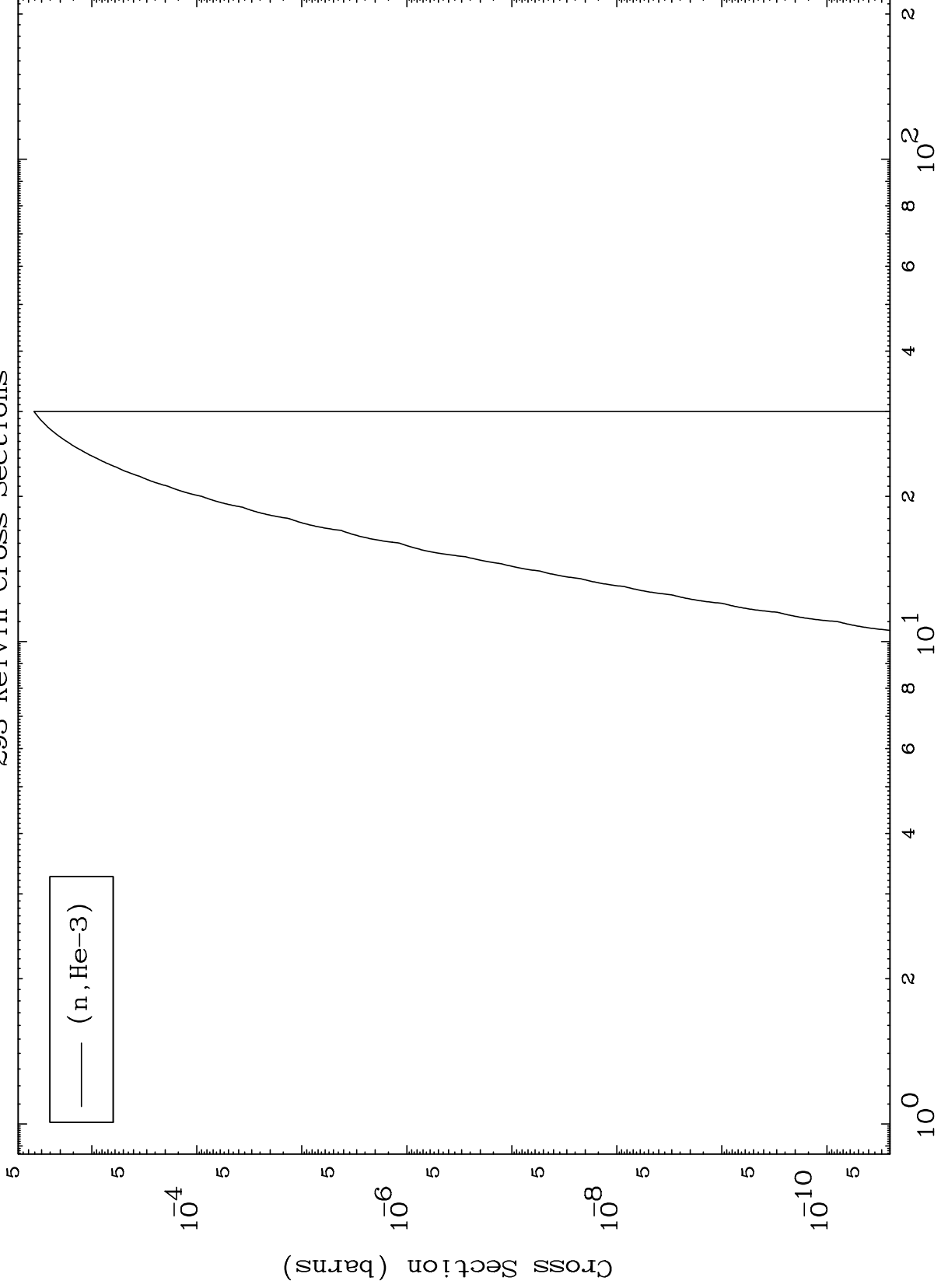
Incident Energy (MeV)

71-Lu-166n

MAT 7100

(n,He3) Levels
293 Kelvin Cross Sections

71-Lu-166n



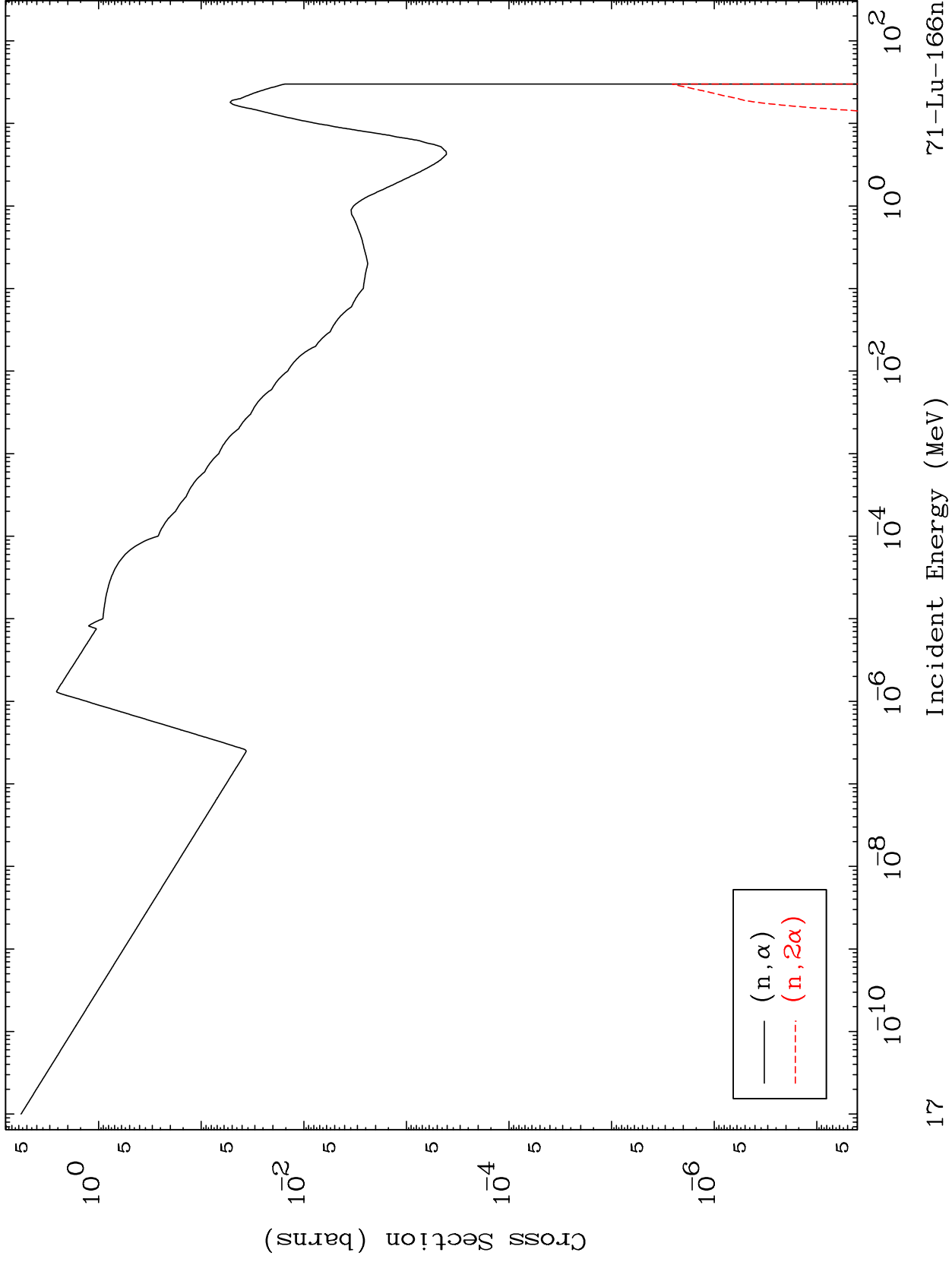
Incident Energy (MeV)

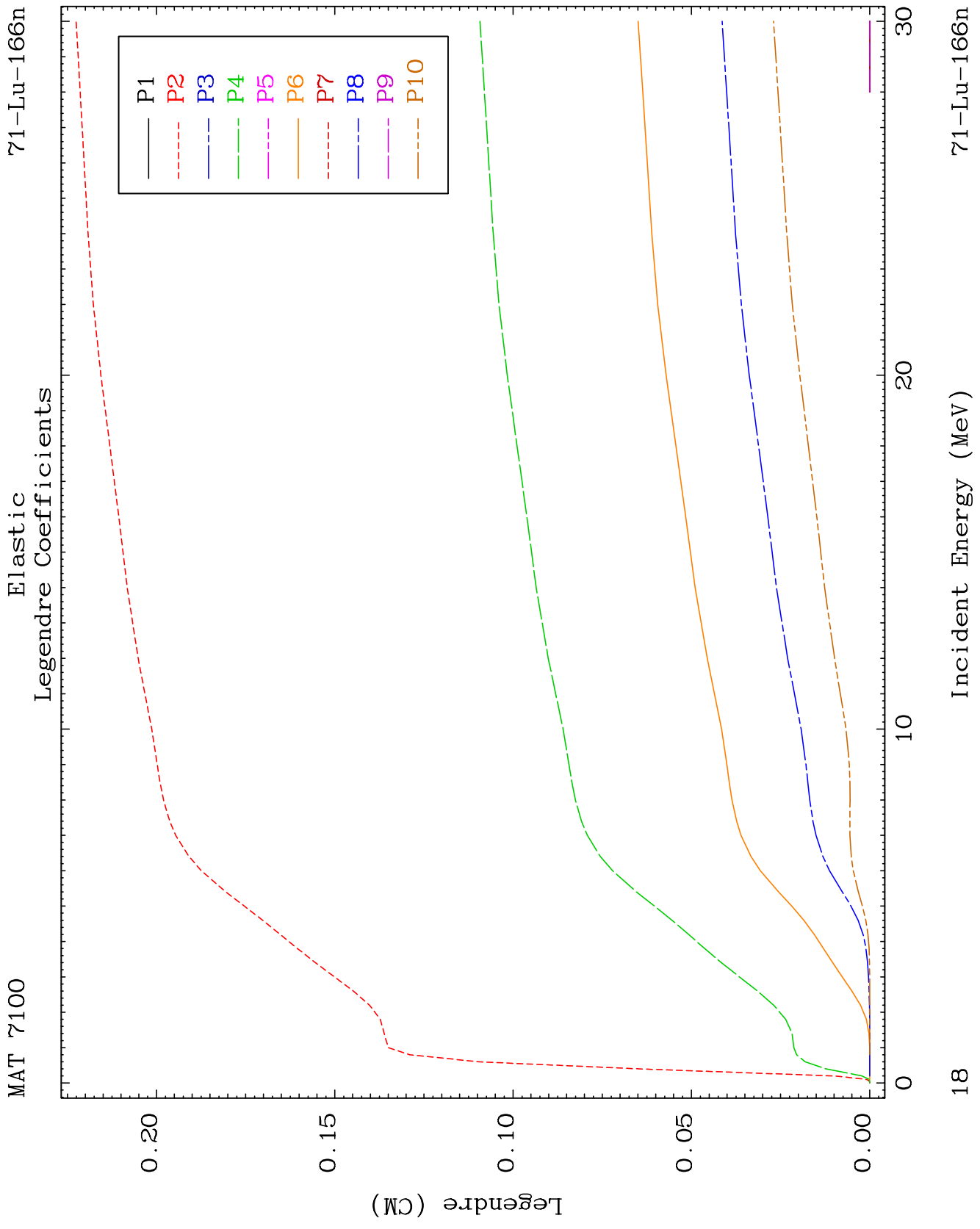
71-Lu-166n

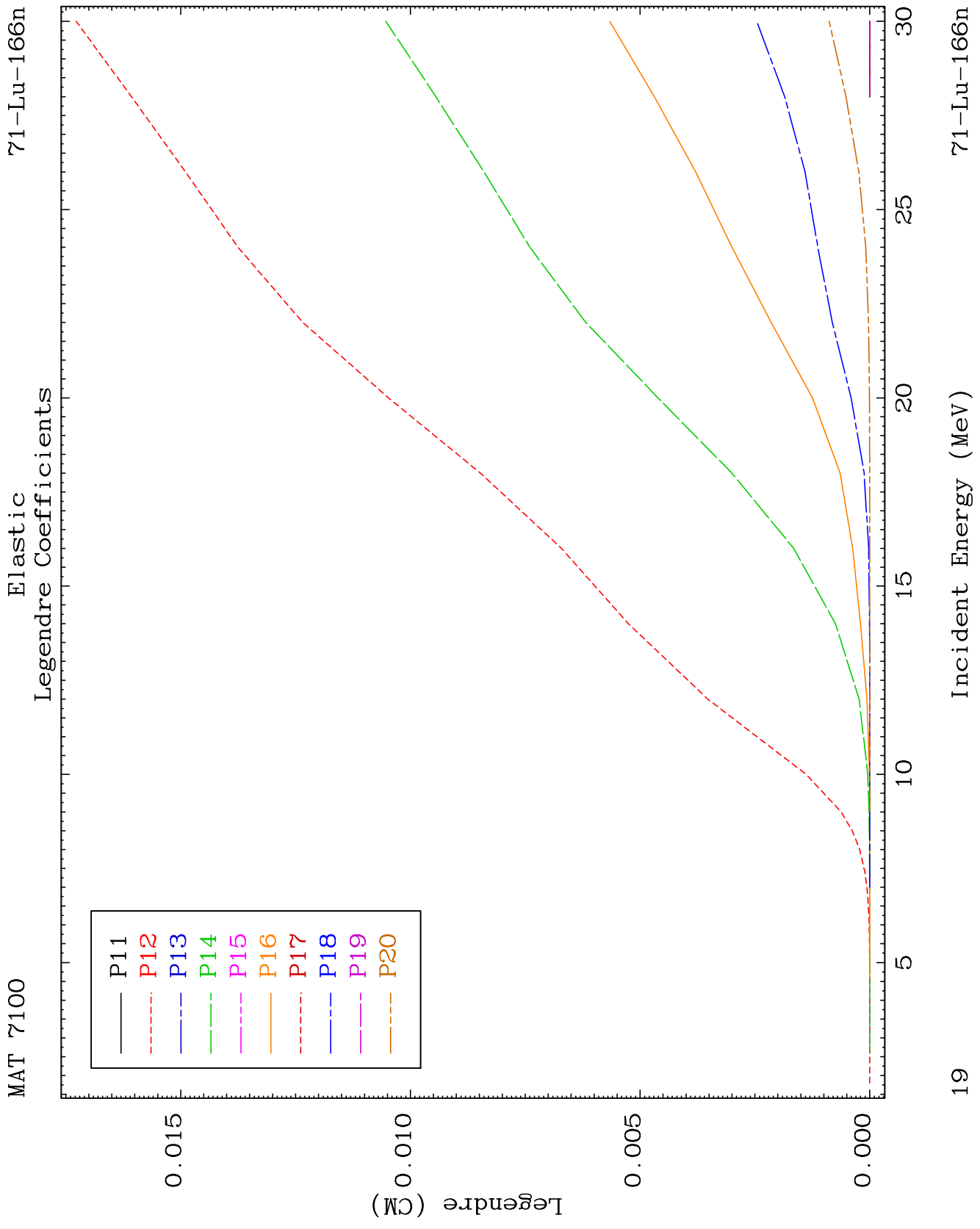
MAT 7100

(n,α) Levels
293 Kelvin Cross Sections

⁷¹Lu-166n



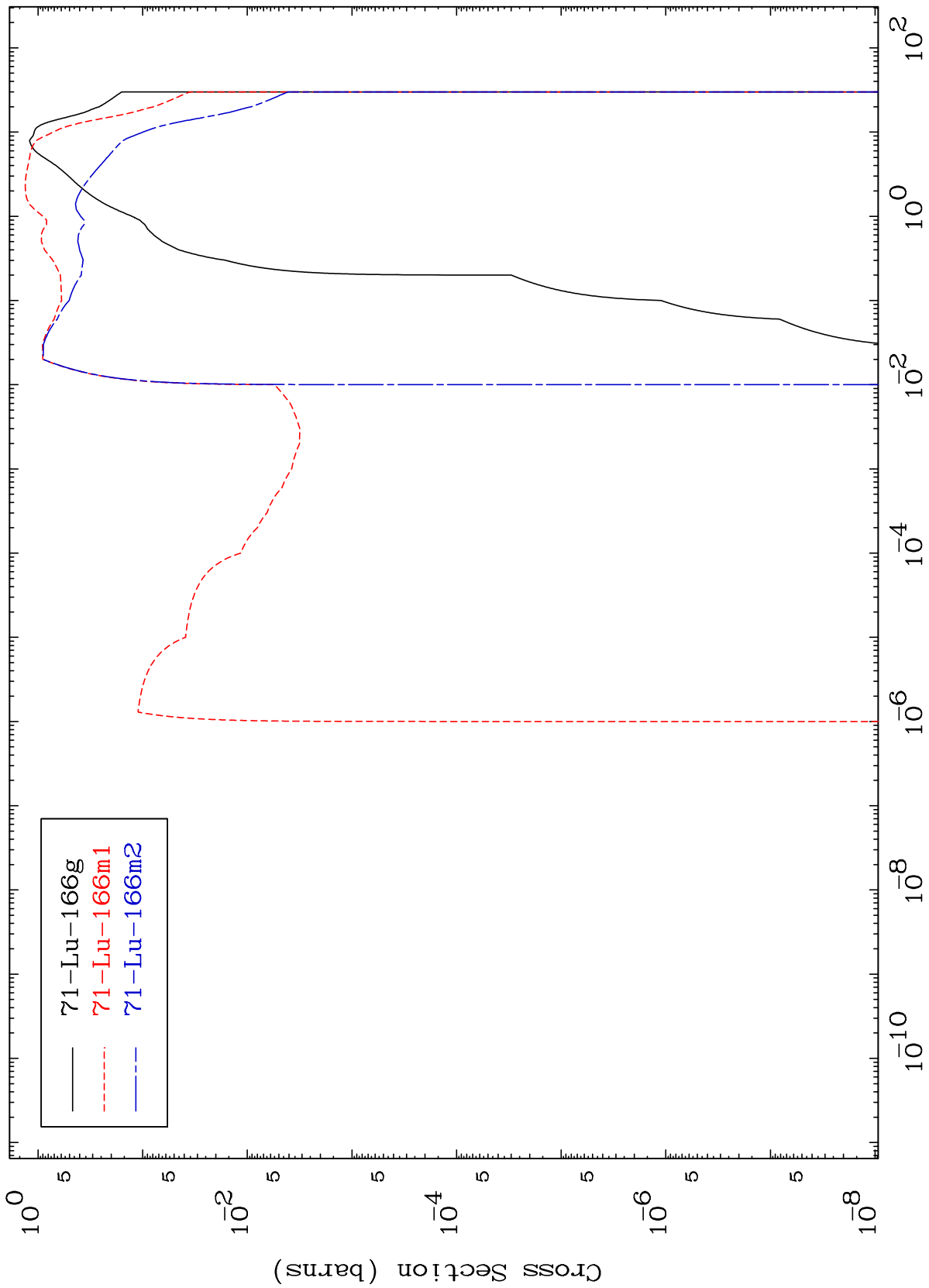




MAT 7100

⁷¹Lu-166n

Inelastic
Radionuclide Production Cross Section



⁷¹Lu-166n

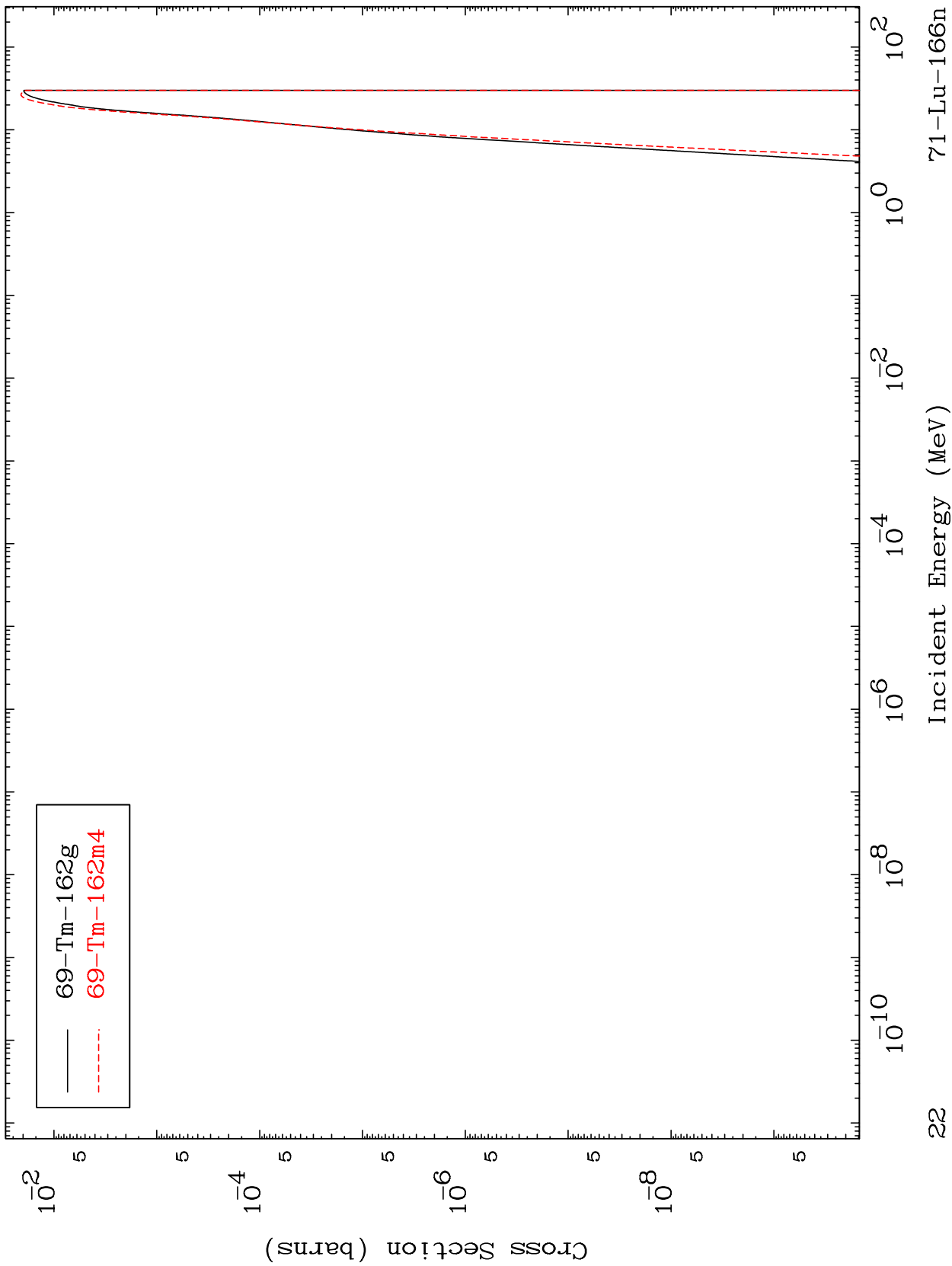
Incident Energy (MeV)

MAT 7100

$(n, n') \alpha$

$^{71}\text{Lu}-166\text{n}$

Radionuclide Production Cross Section



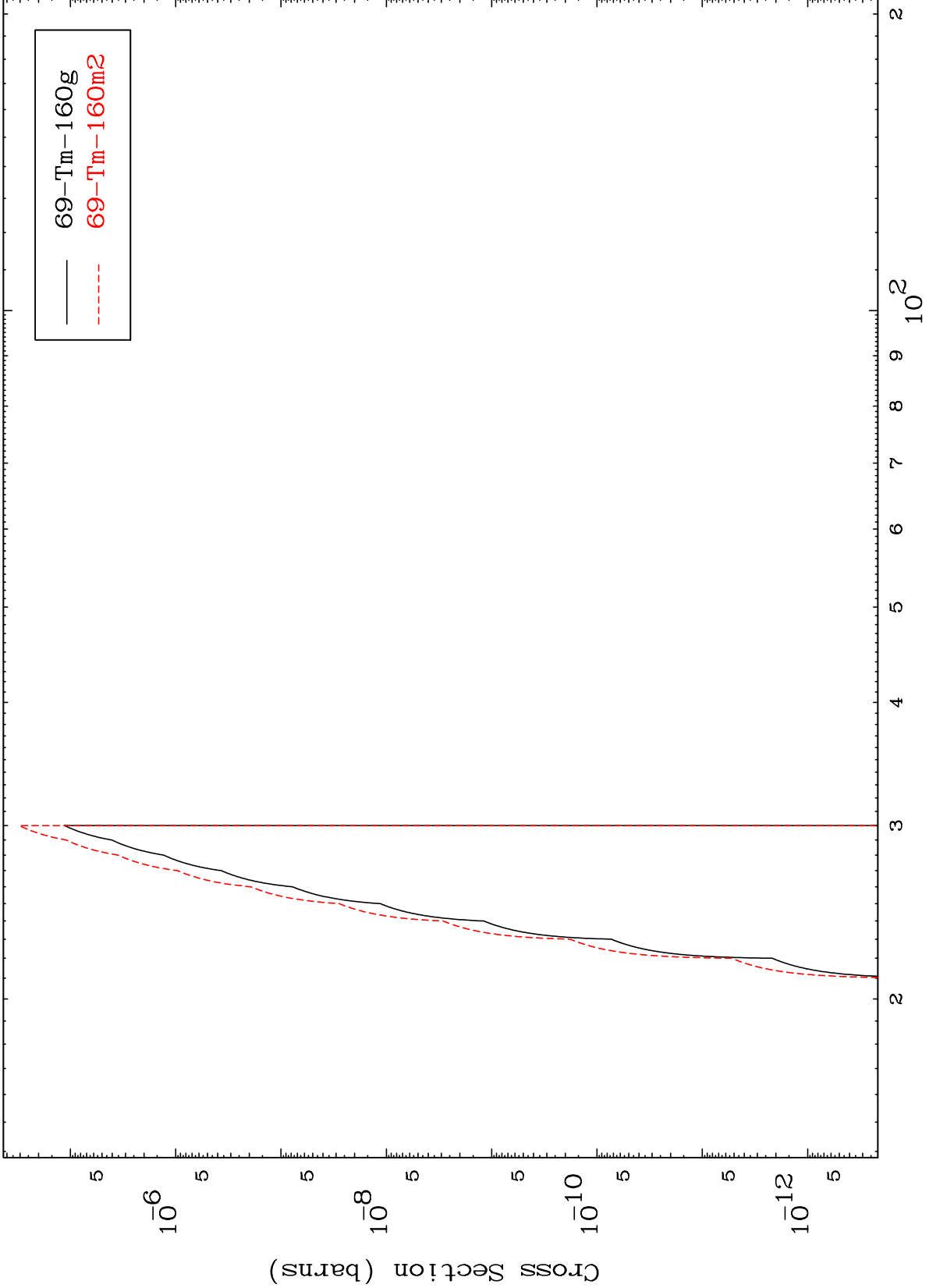
69-Tm-162g
69-Tm-162m4

MAT 7100

(n,3n) α

⁷¹Lu-166n

Radionuclide Production Cross Section



23

Incident Energy (MeV)

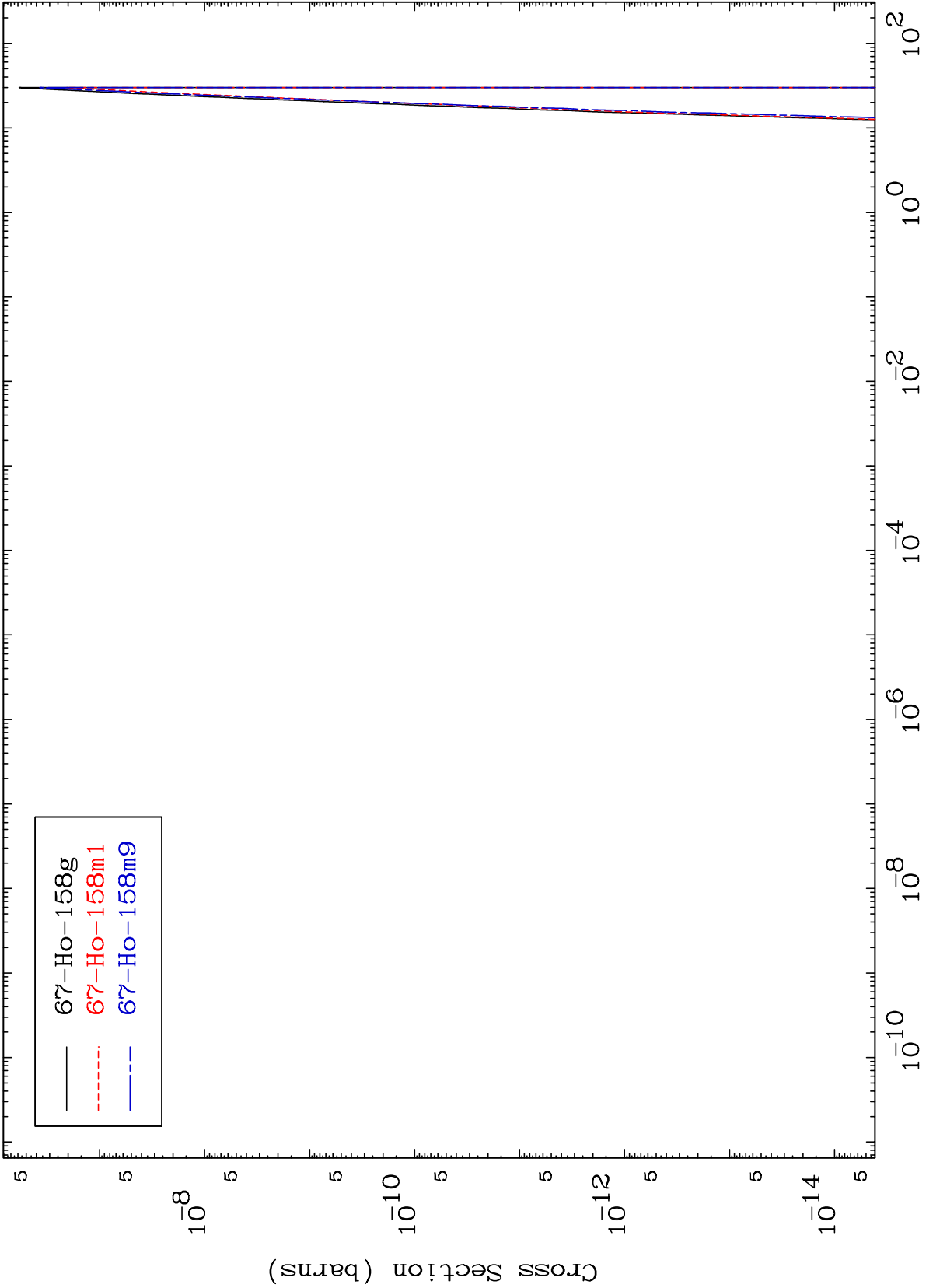
⁷¹Lu-166n

MAT 7100

(n,n') 2α

71-Lu-166n

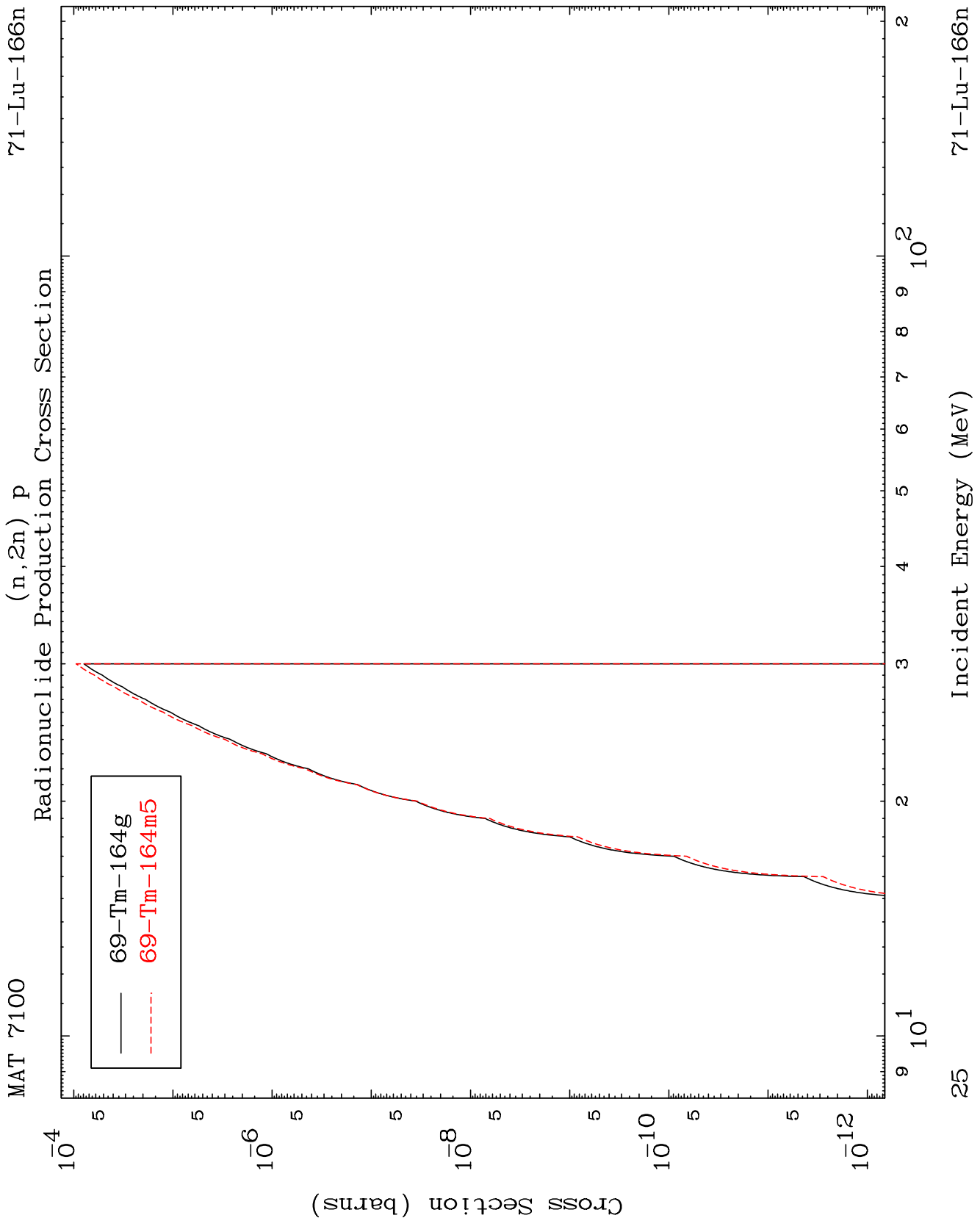
Radionuclide Production Cross Section



24

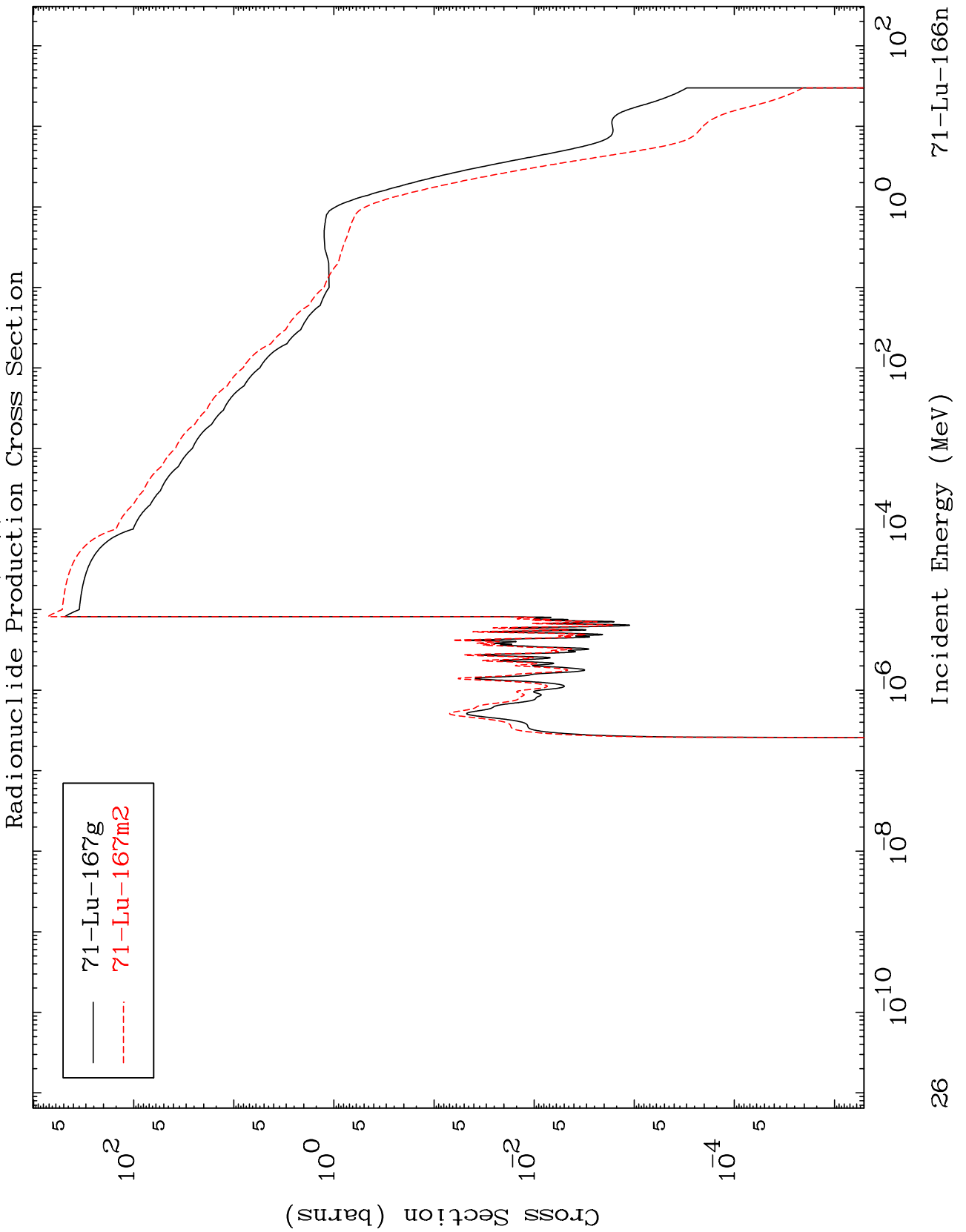
Incident Energy (MeV)

71-Lu-166n



MAT 7100

⁷¹Lu-166n

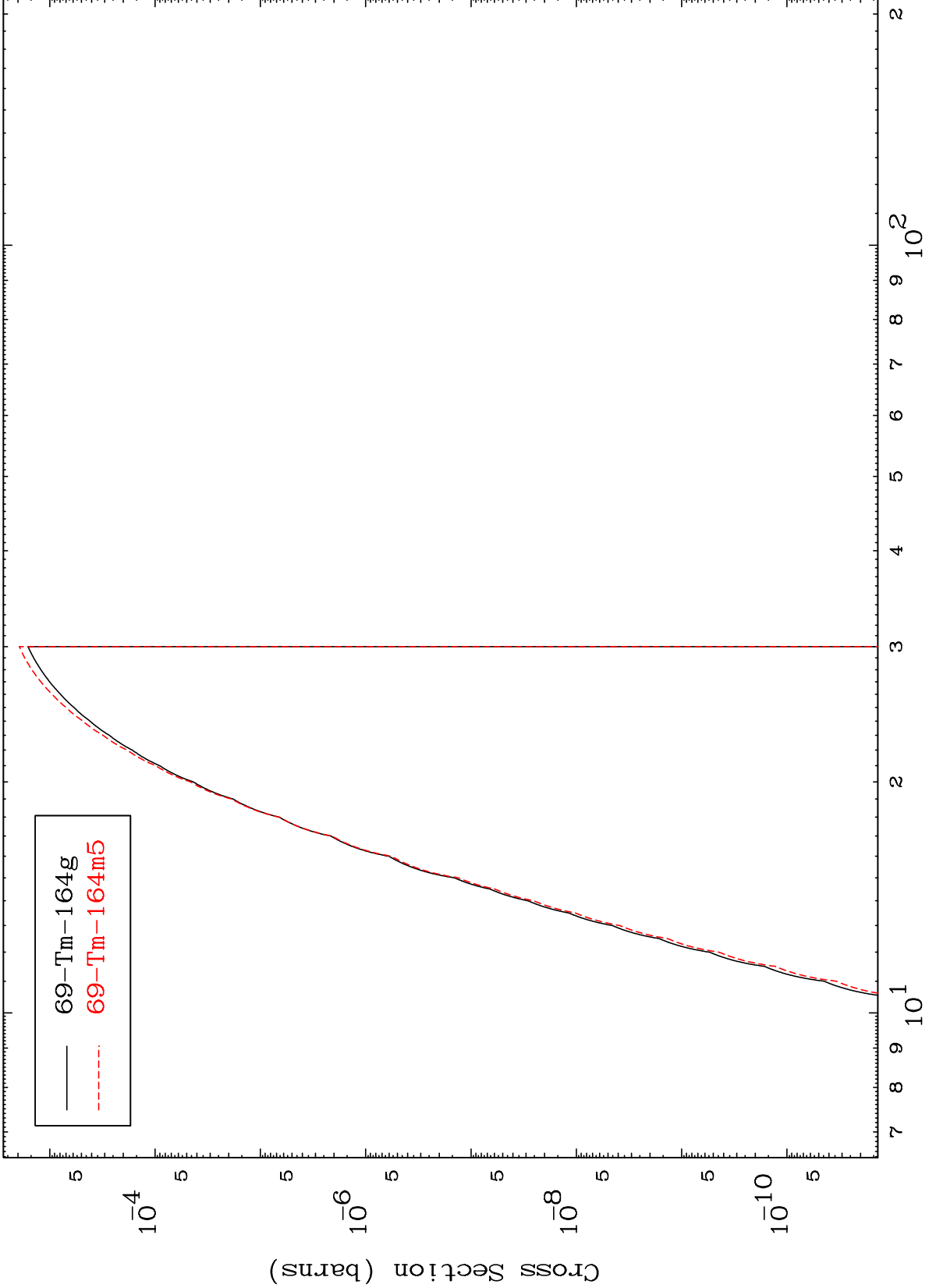


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(n,He-3)

⁷¹Lu-166n

Radionuclide Production Cross Section



69-Tm-164g
69-Tm-164m5

27

Incident Energy (MeV)

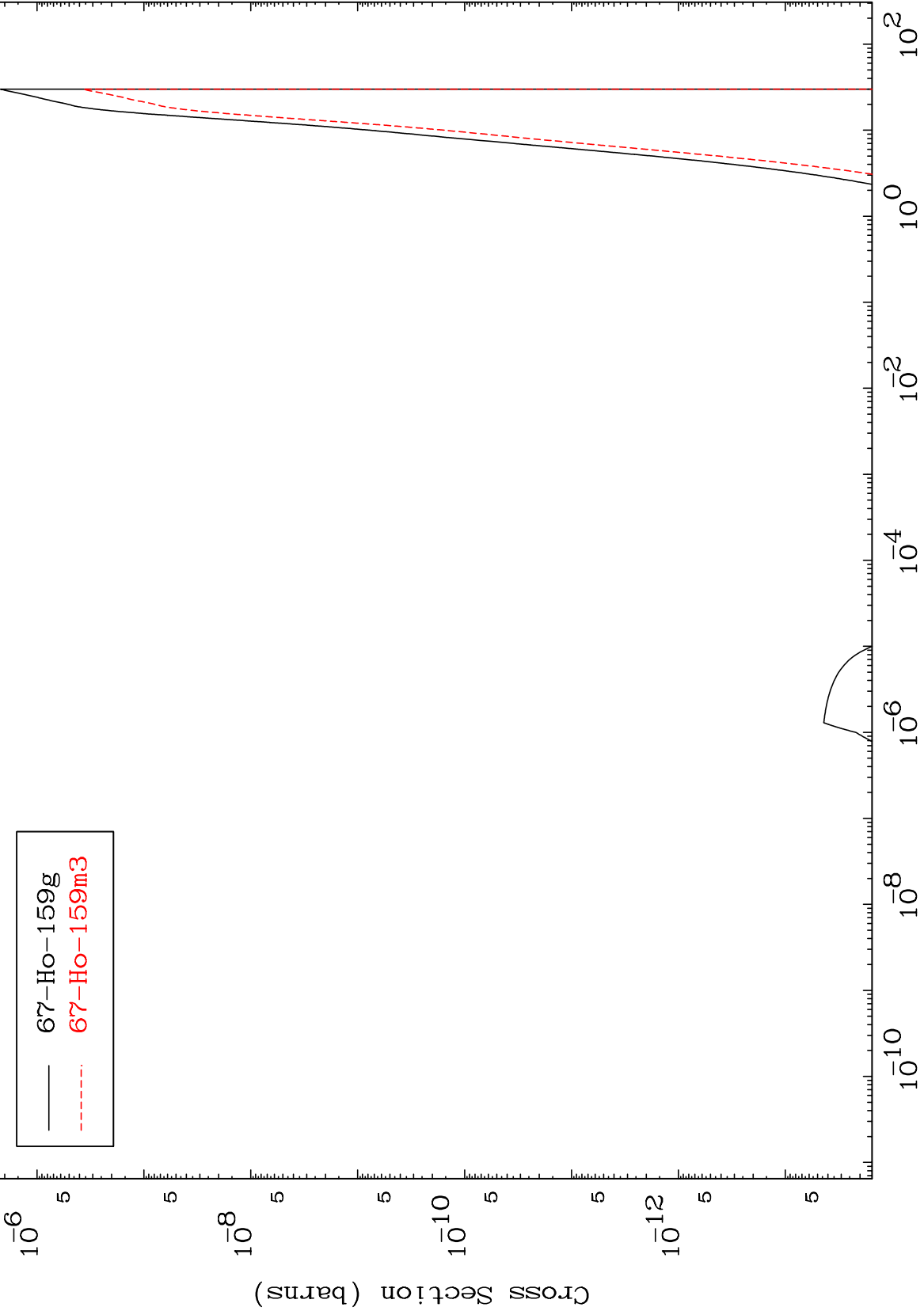
⁷¹Lu-166n

MAT 7100

(n,2α)

⁷¹Lu-166n

Radionuclide Production Cross Section

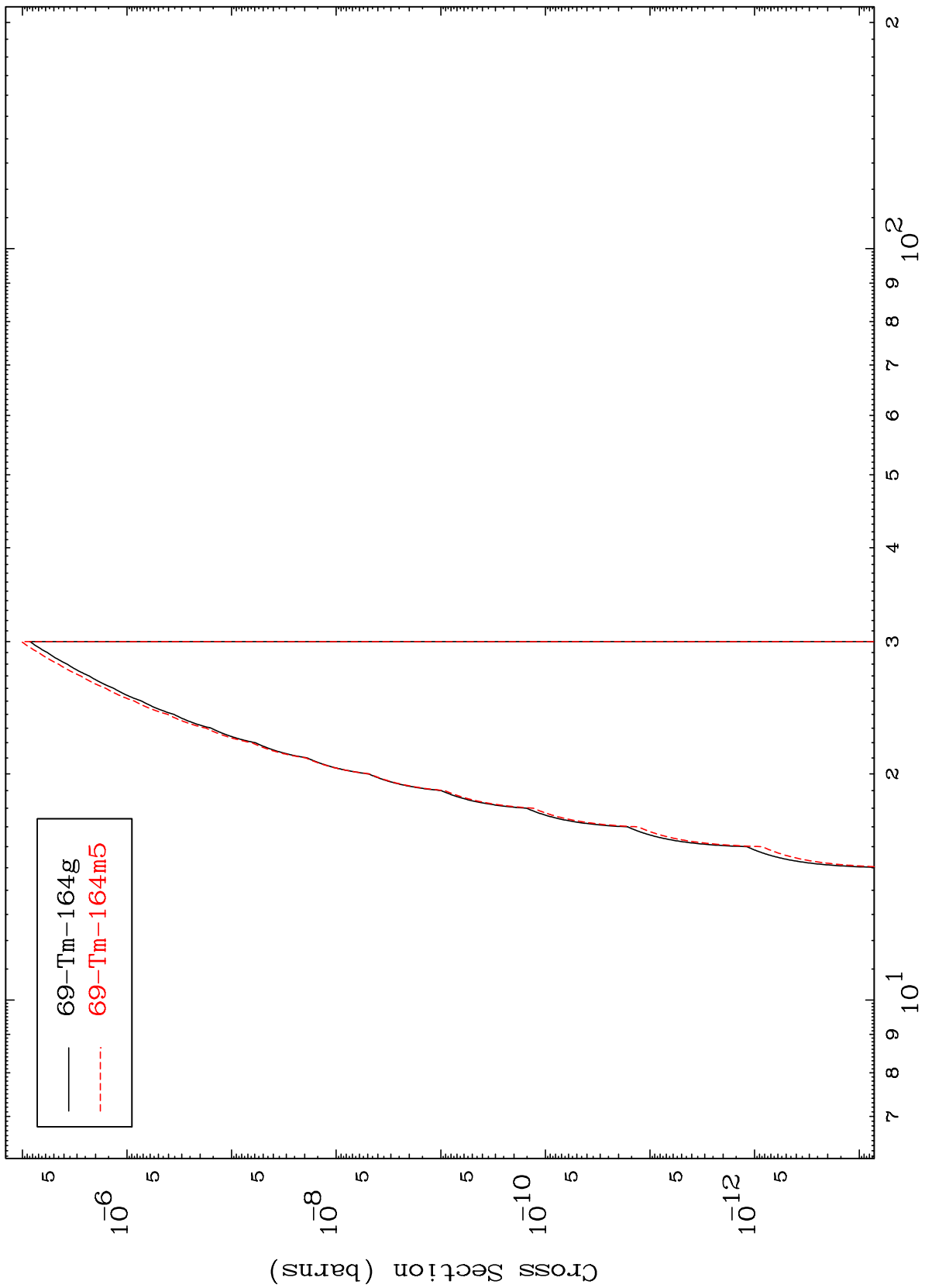


MAT 7100

(n,p) d

⁷¹Lu-166n

Radionuclide Production Cross Section



29

Incident Energy (MeV)

⁷¹Lu-166n