

Program EVALPLOT
(Version 2021-1)

by

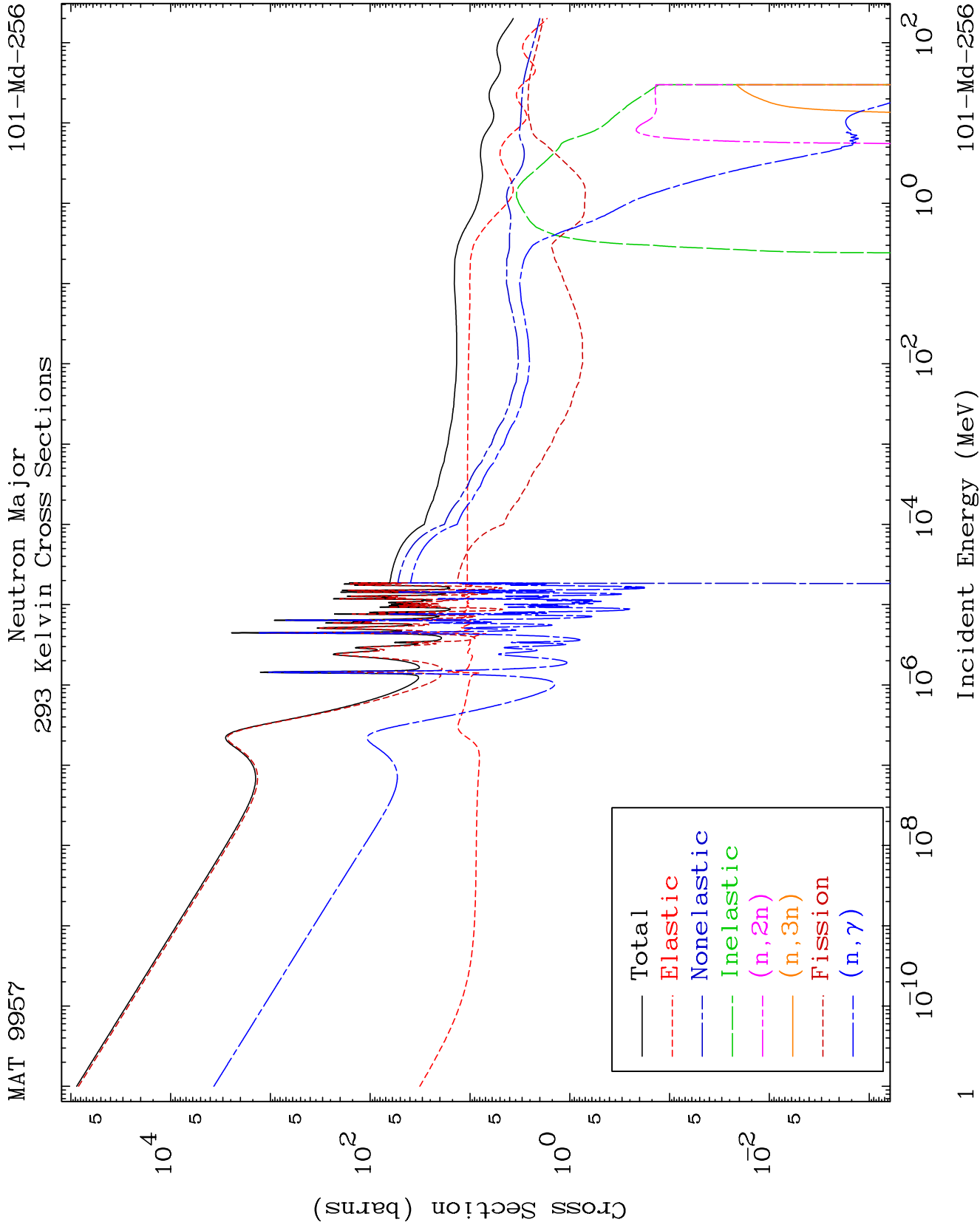
Dermott E. Cullen
(Present Contact Information)

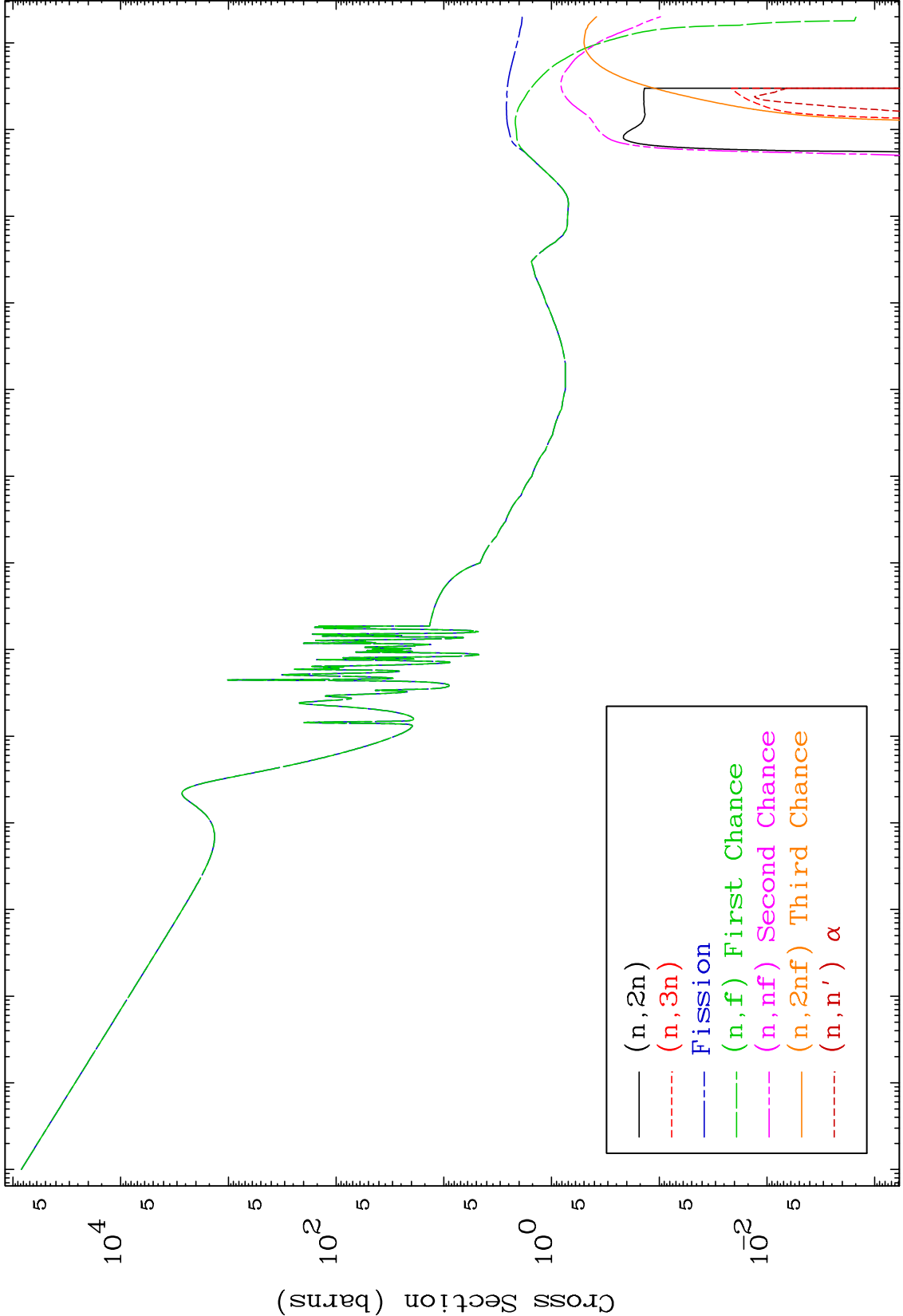
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

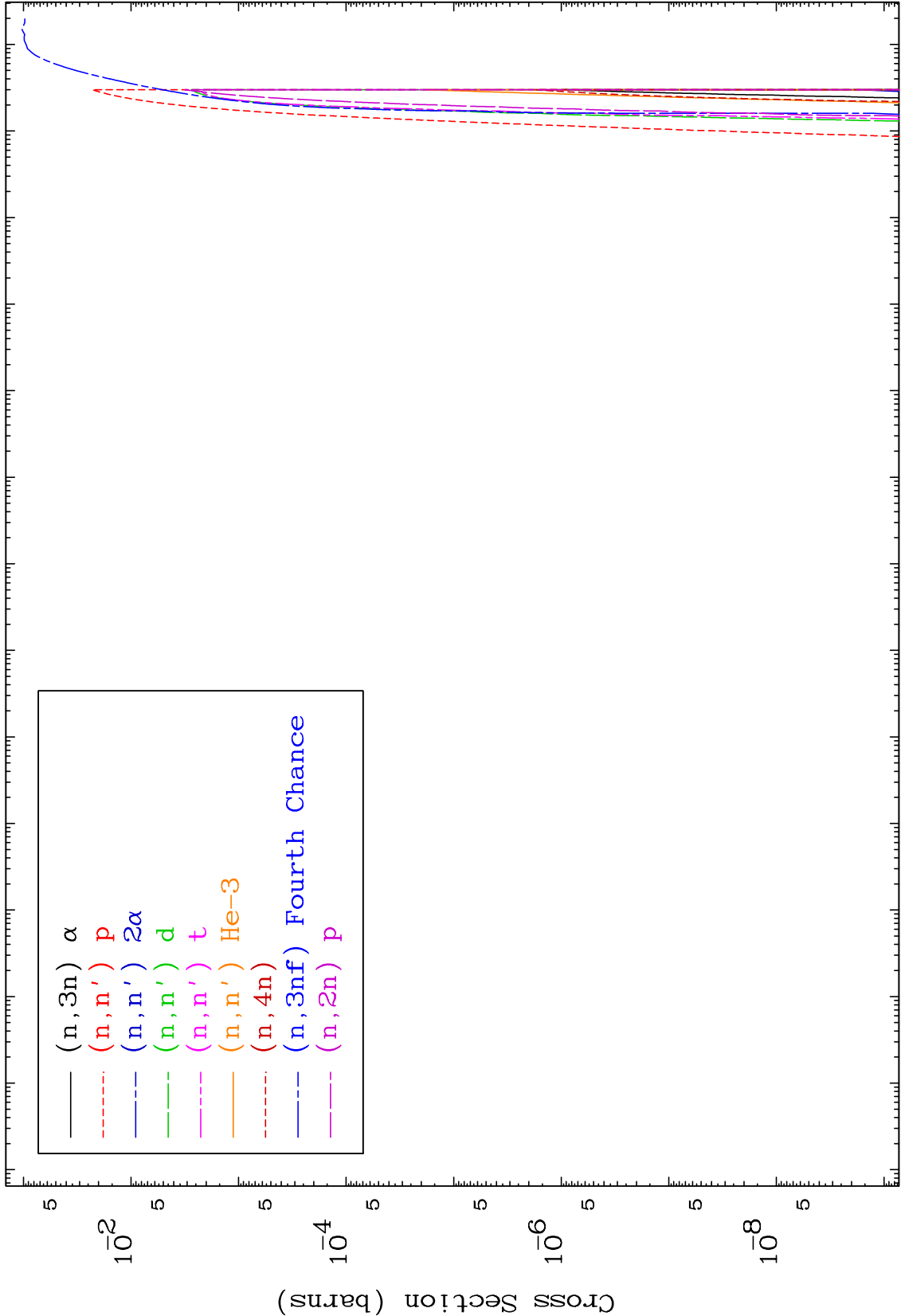


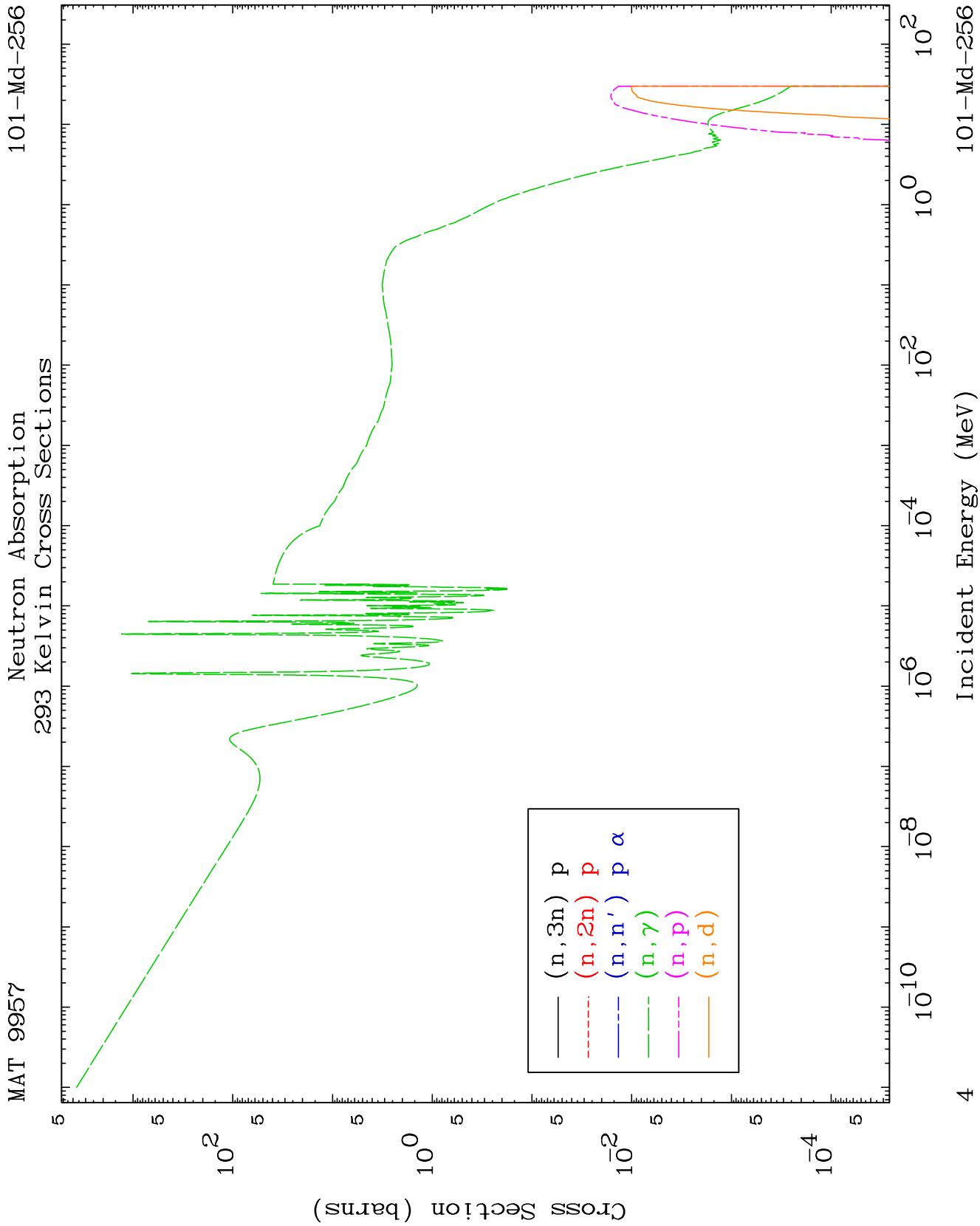


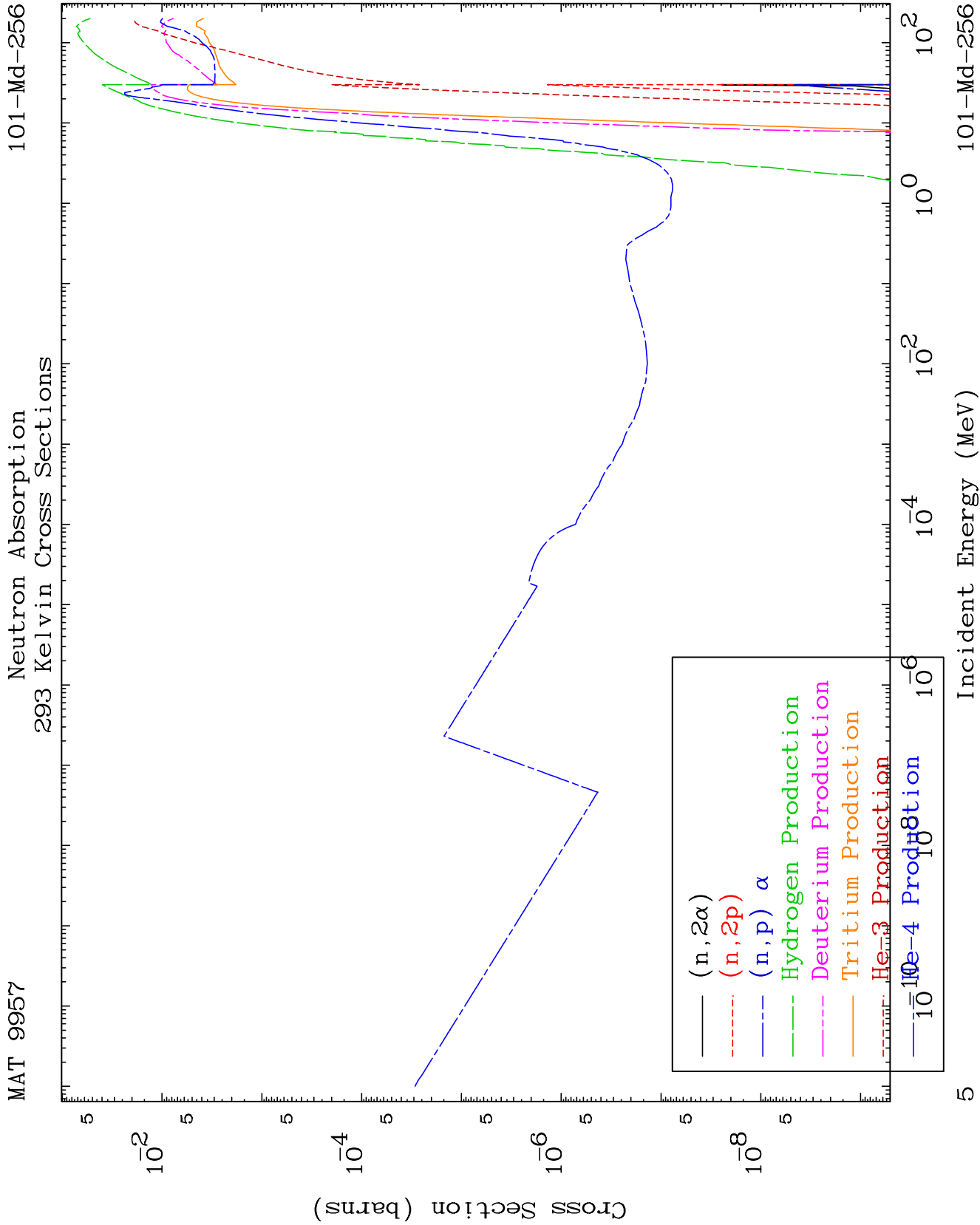
MAT 9957

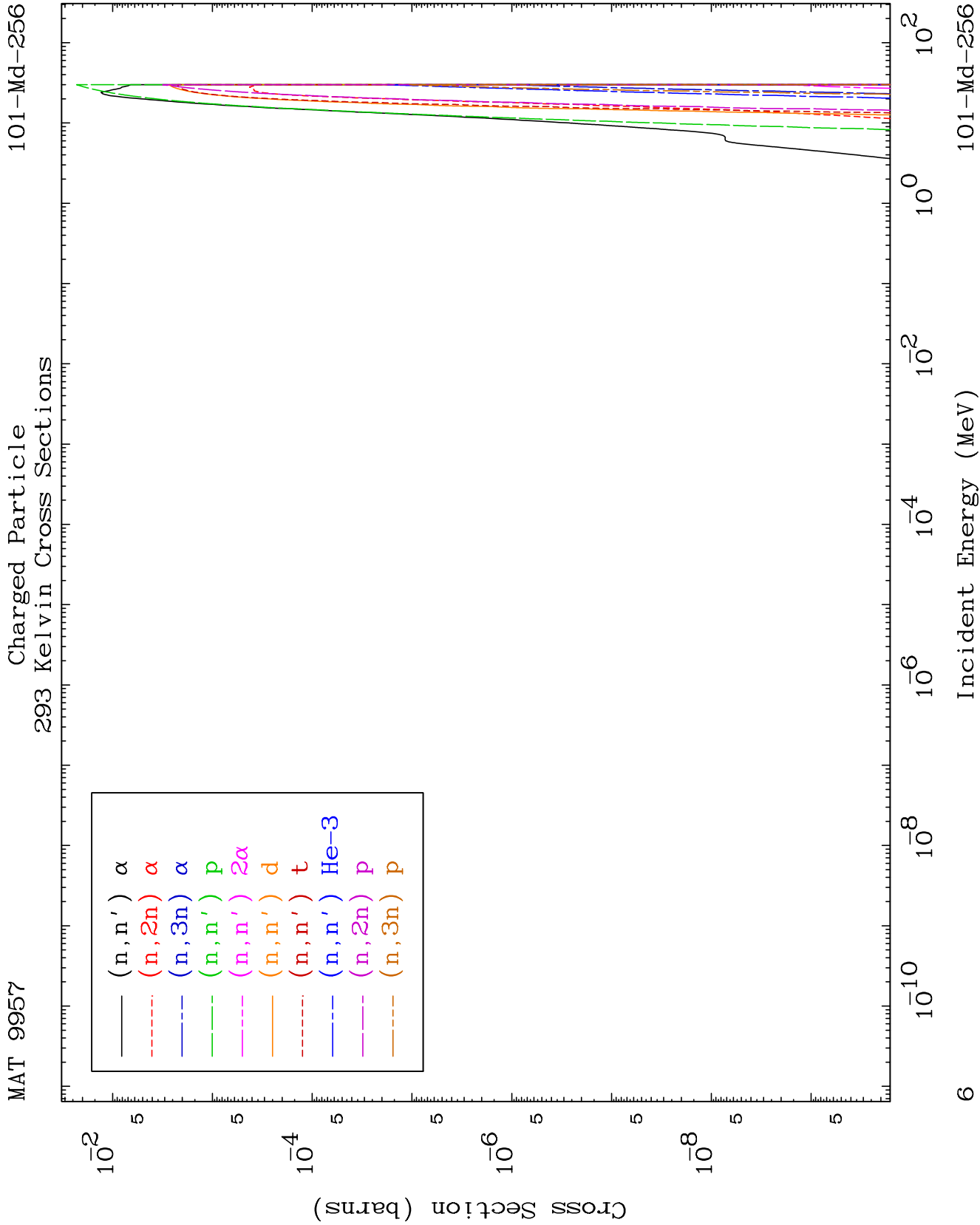
Neutron Absorption
293 Kelvin Cross Sections

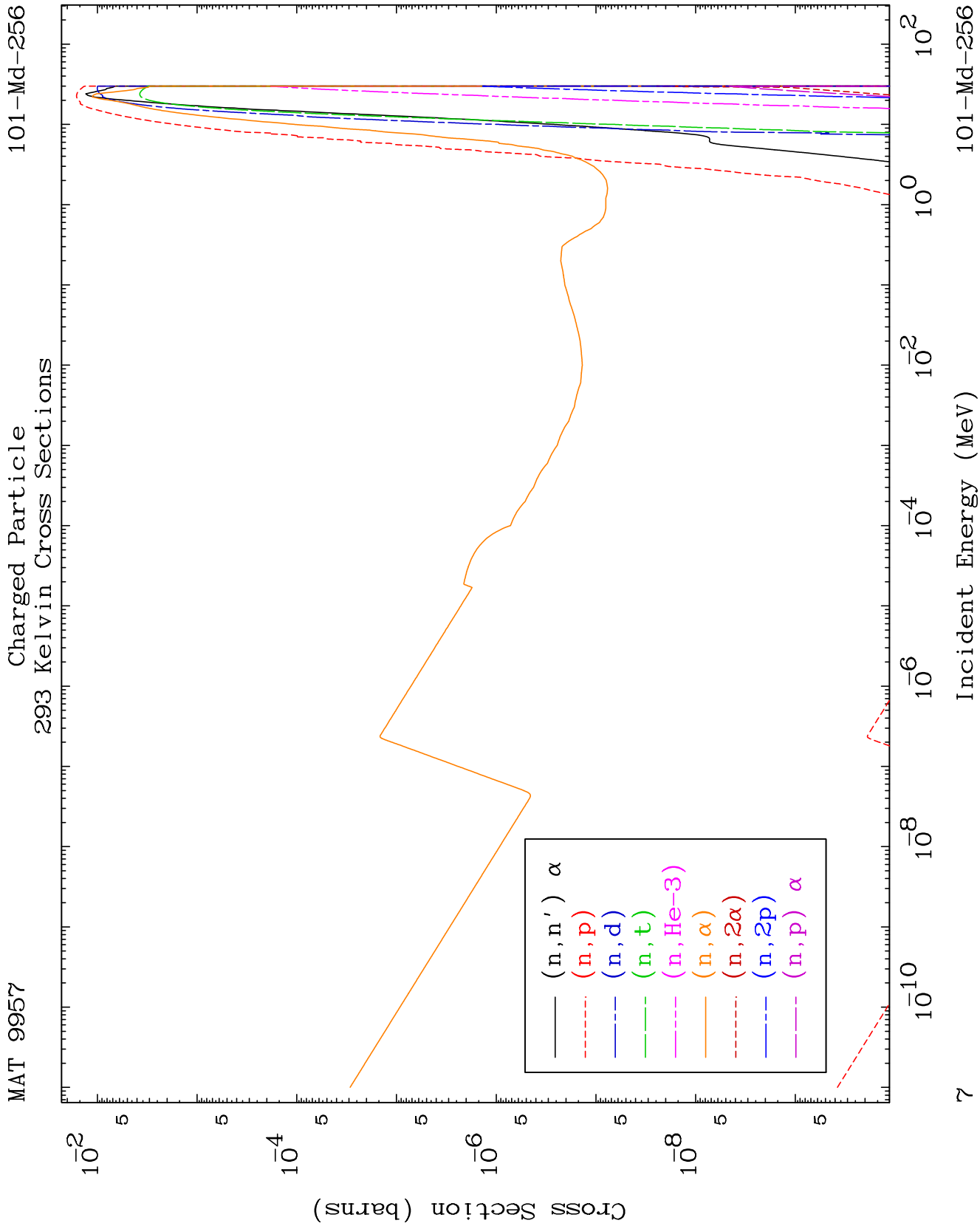
101-Md-256







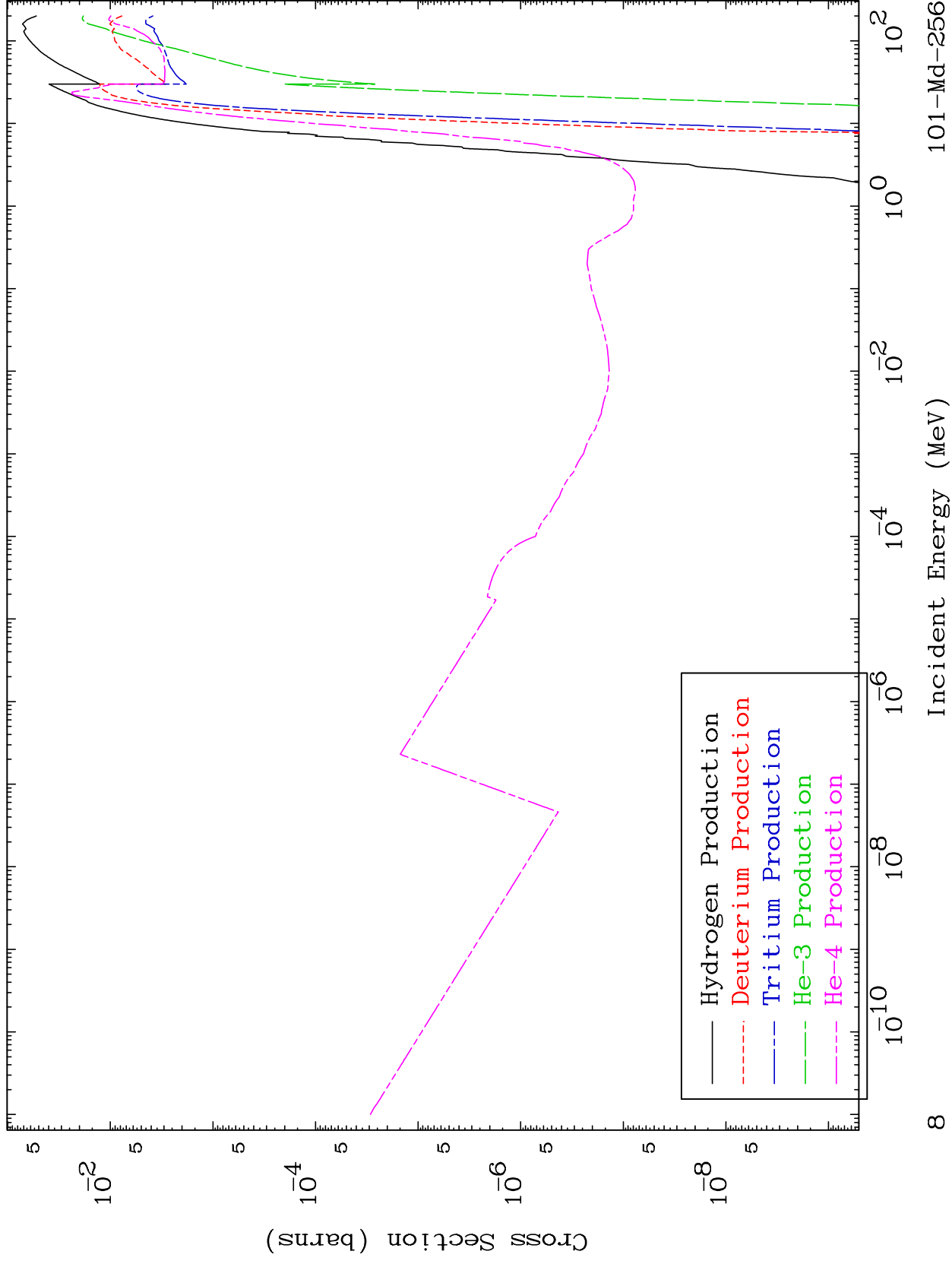


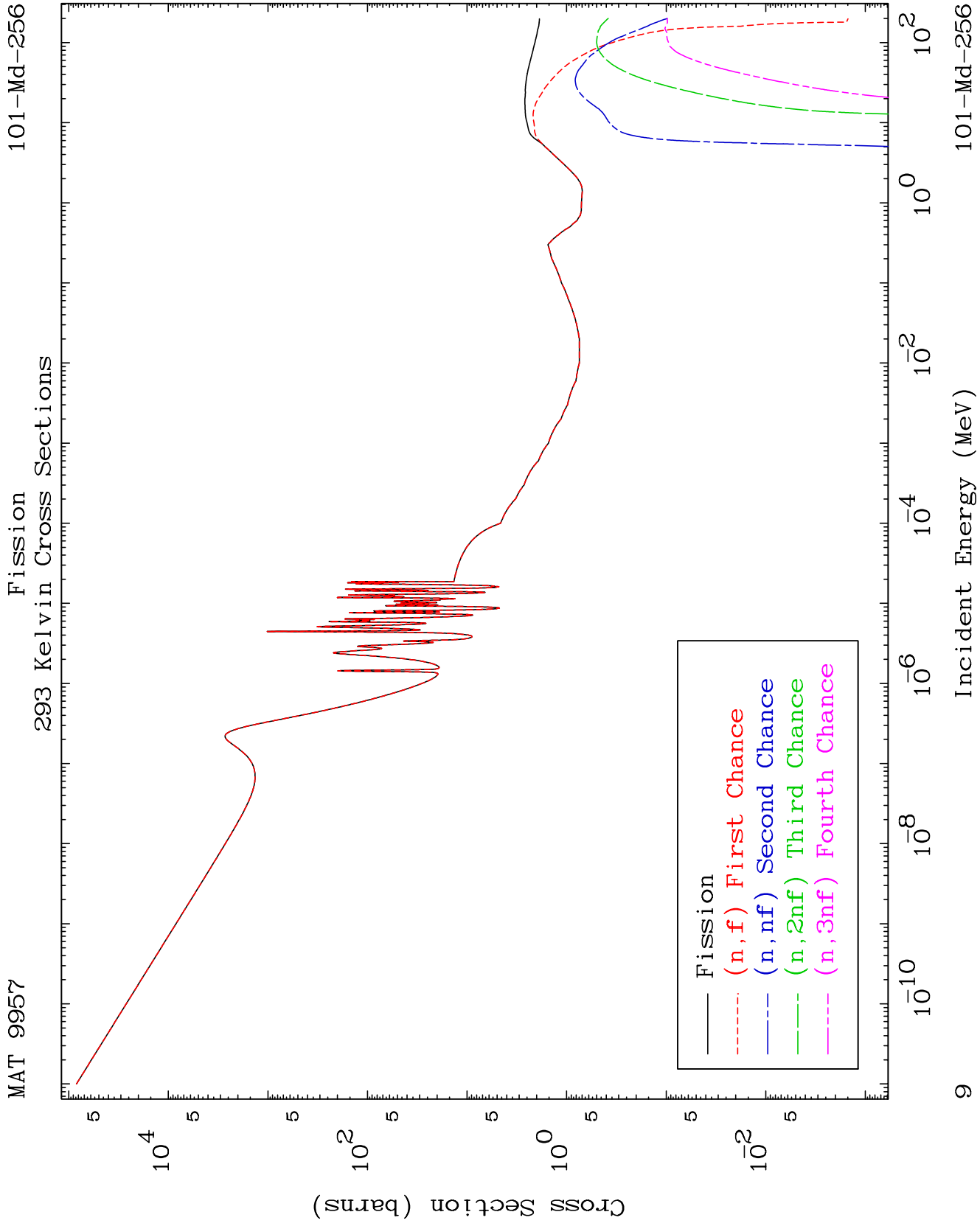


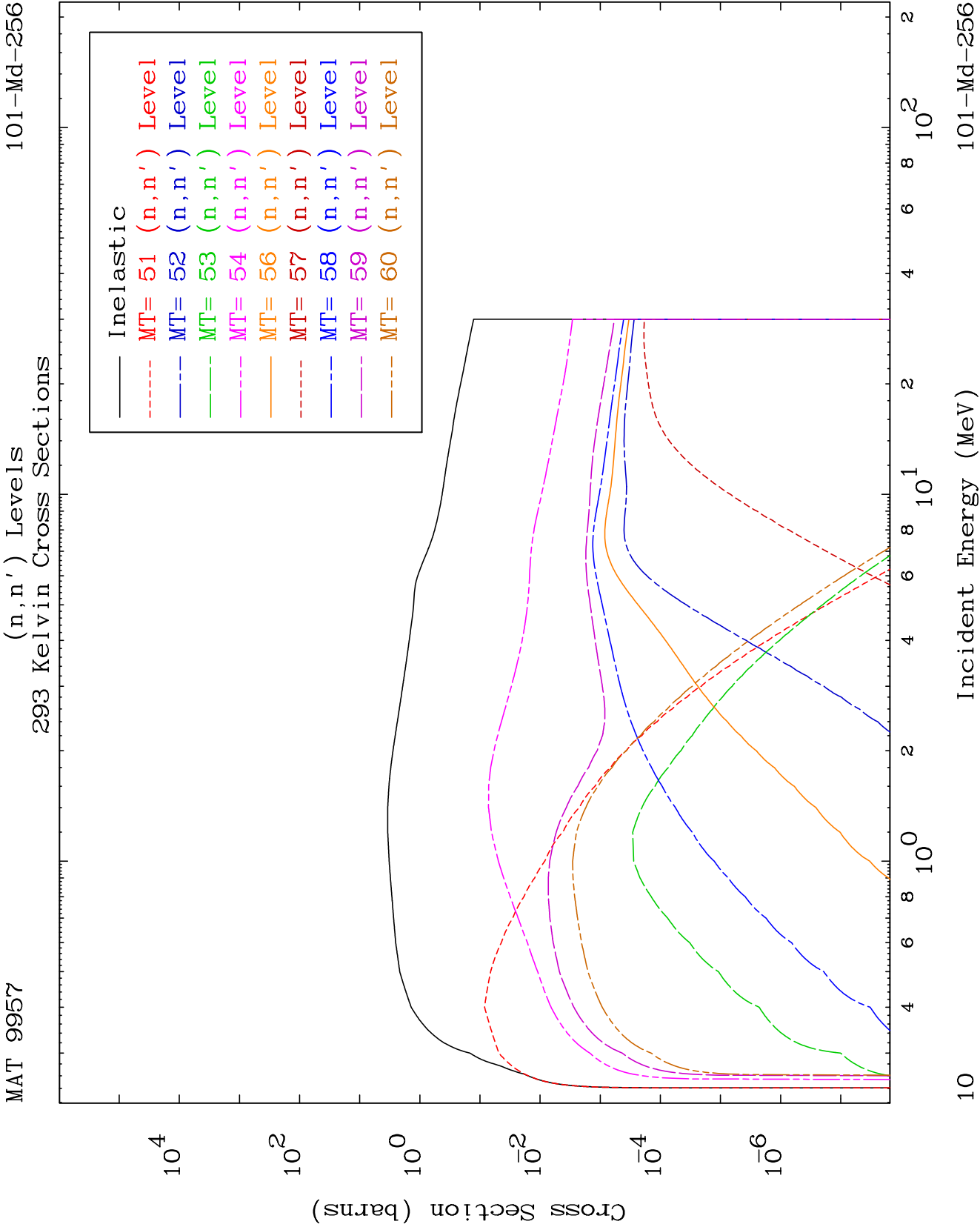
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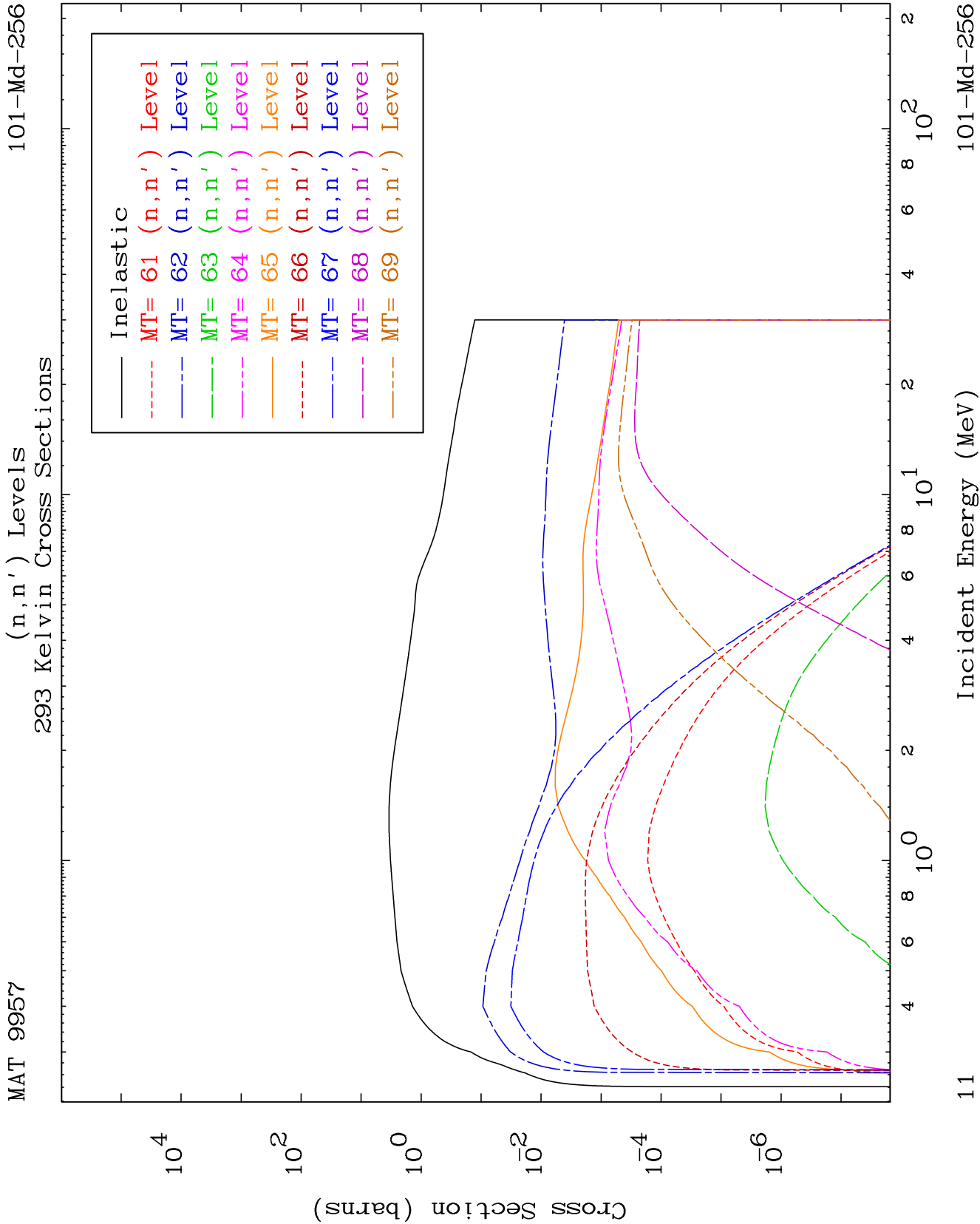
Particle Production
293 Kelvin Cross Sections

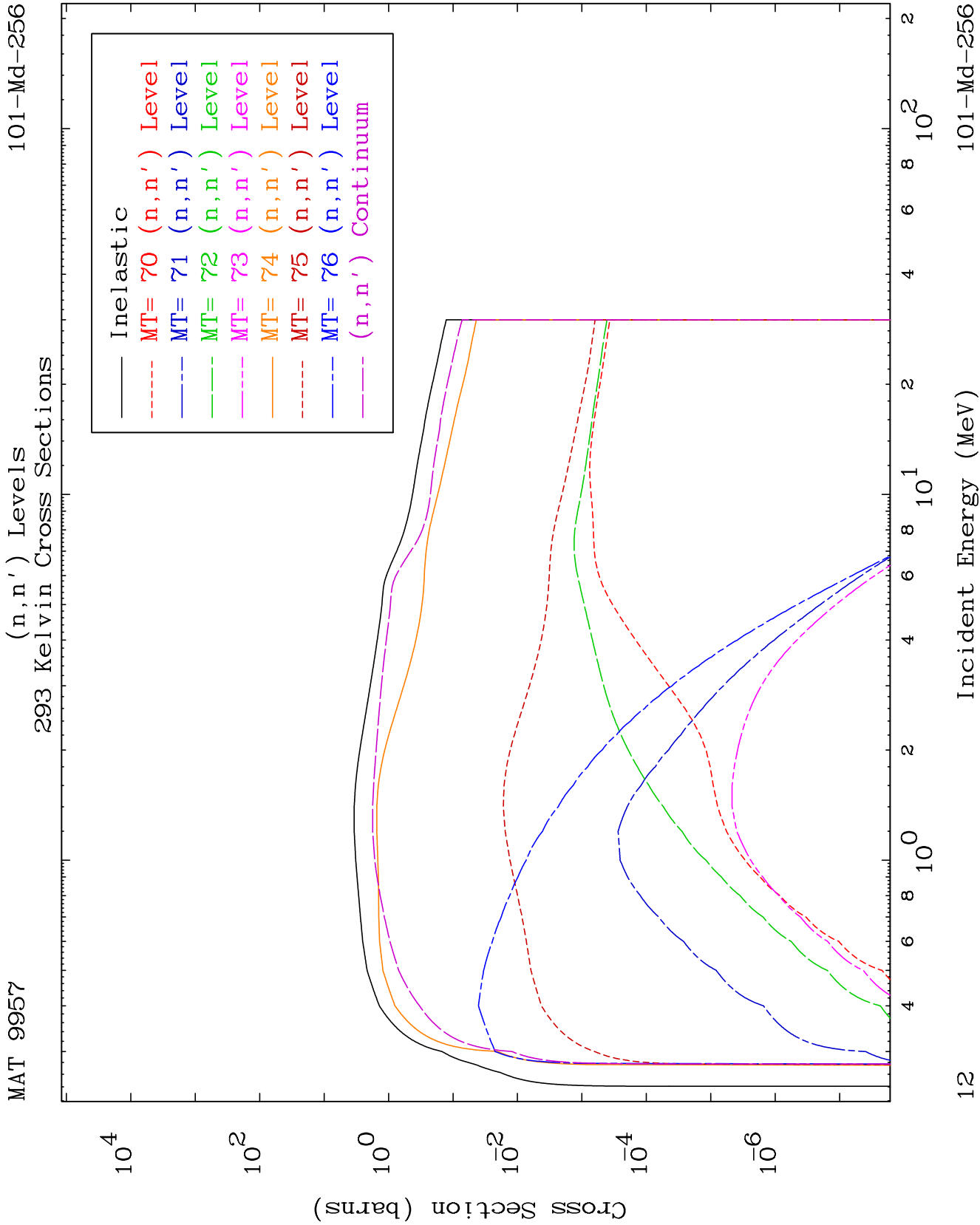
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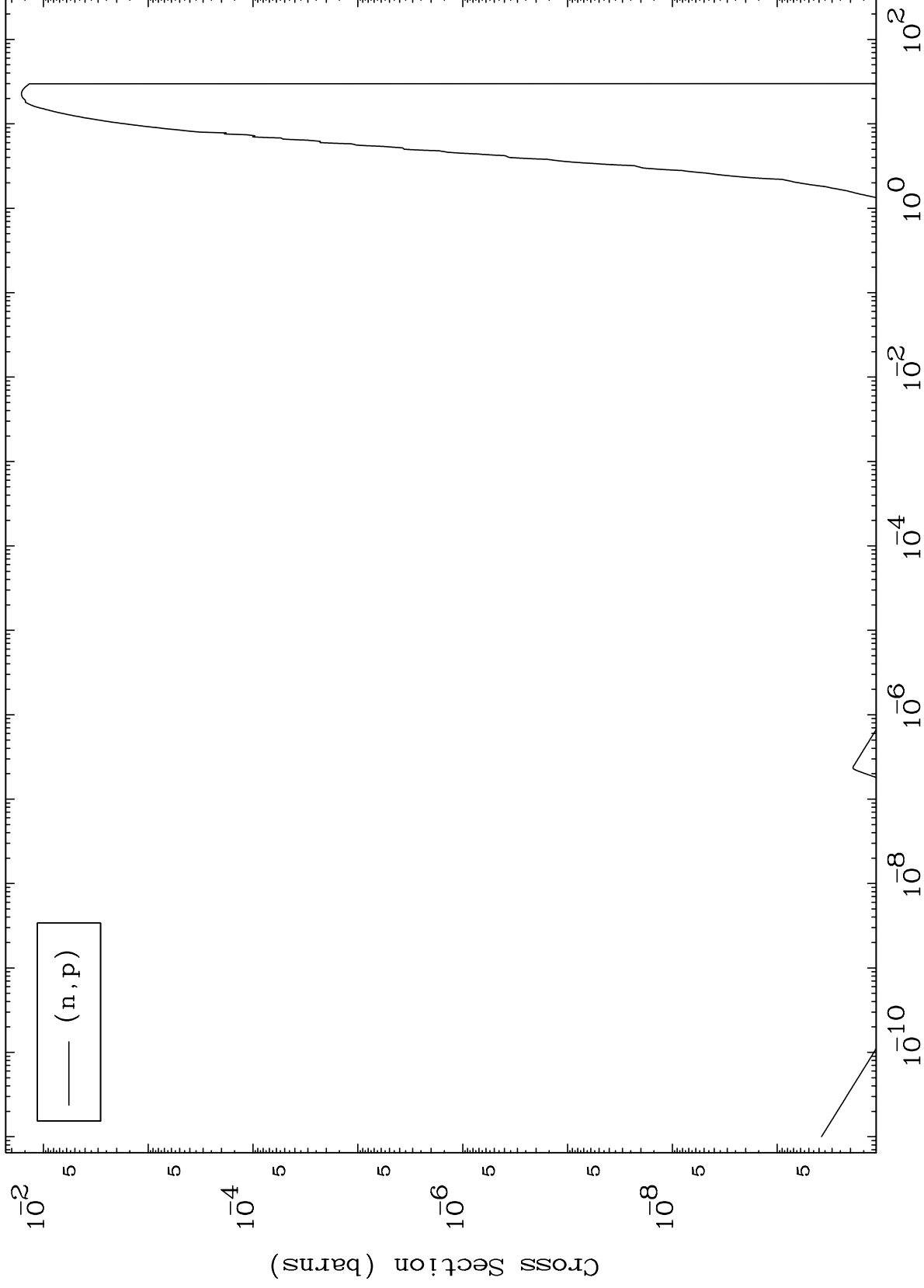




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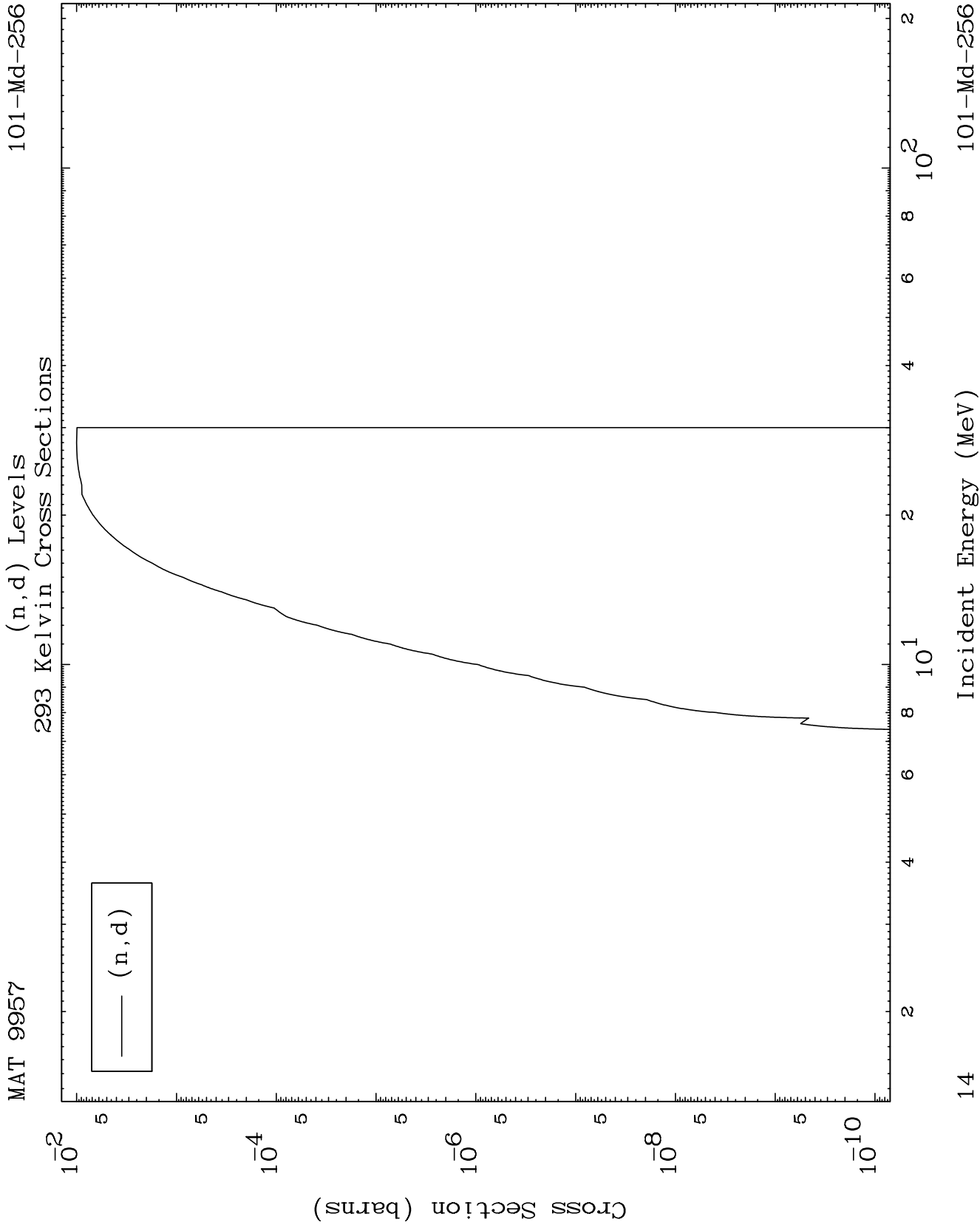
(n,p) Levels
293 Kelvin Cross Sections

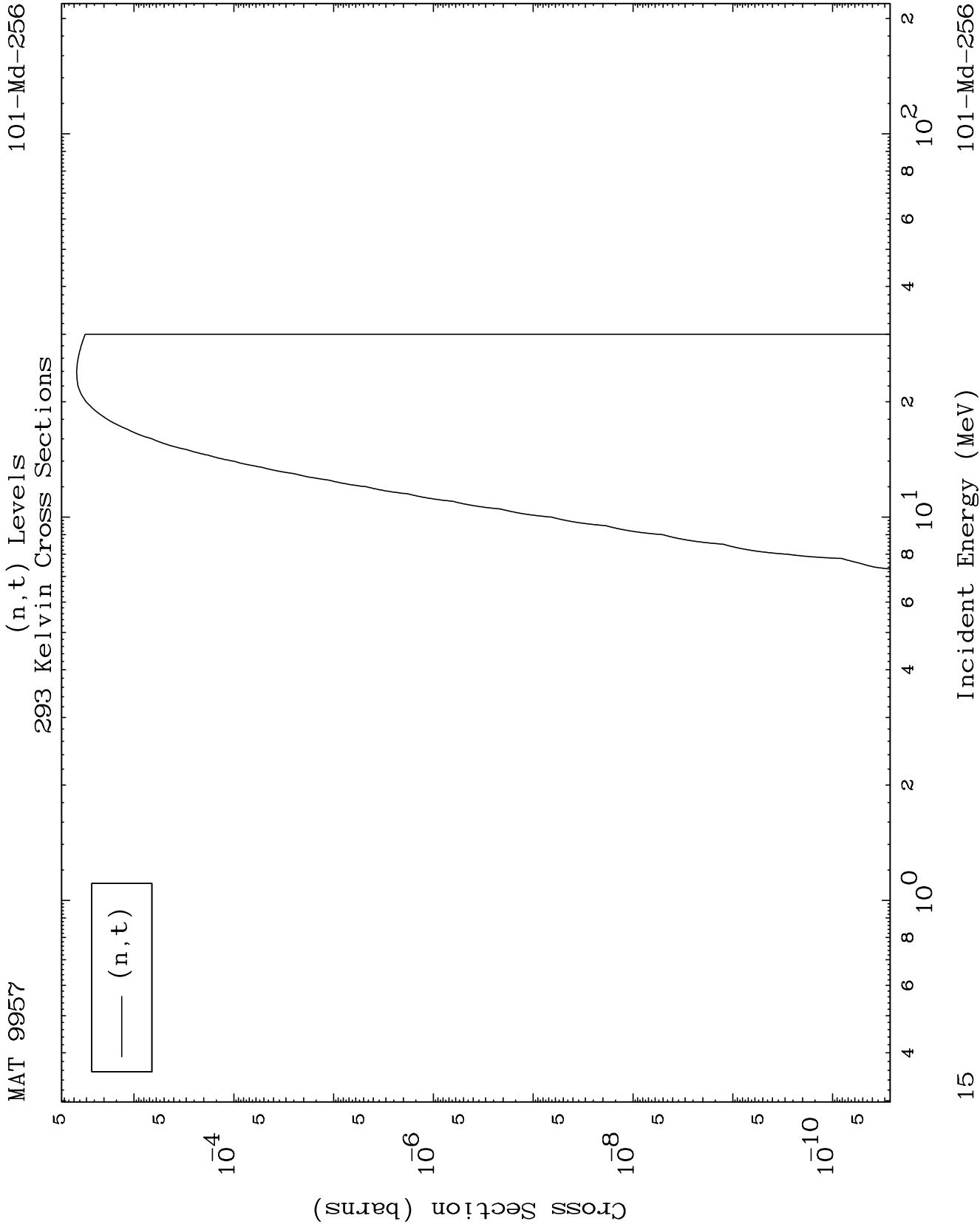


13

101-Md-256

Incident Energy (MeV)

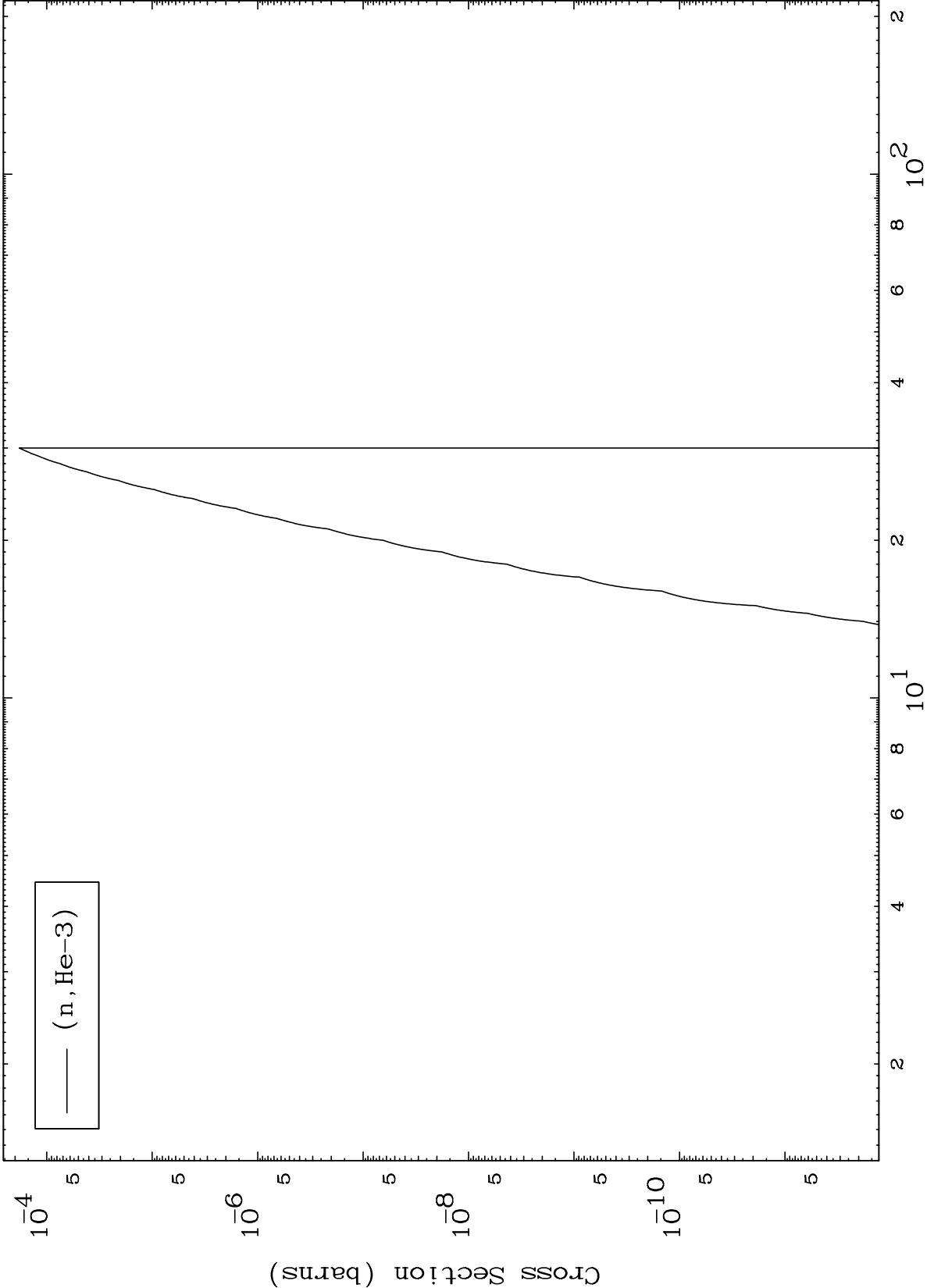




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(n,He3) Levels
293 Kelvin Cross Sections

101-Md-256

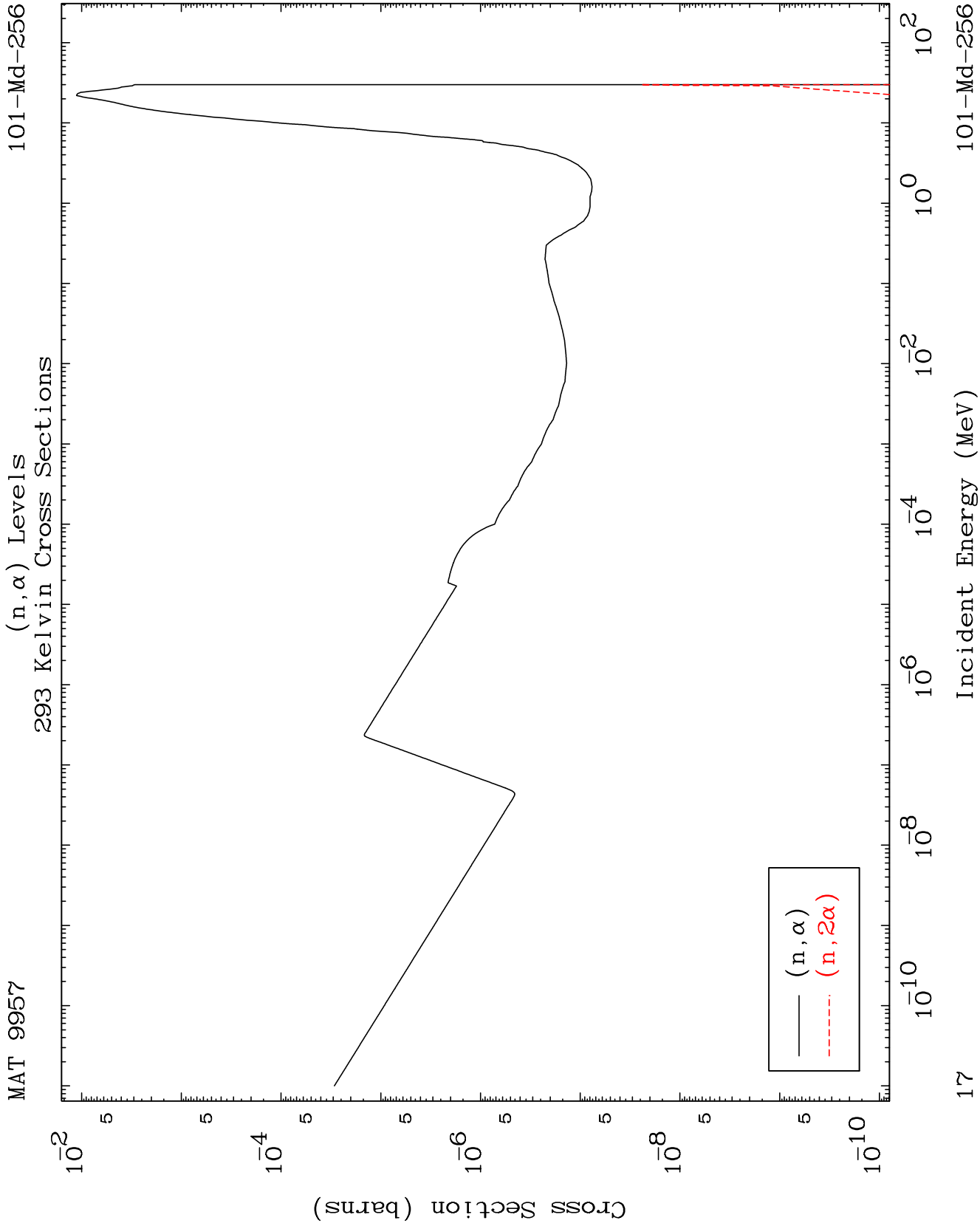


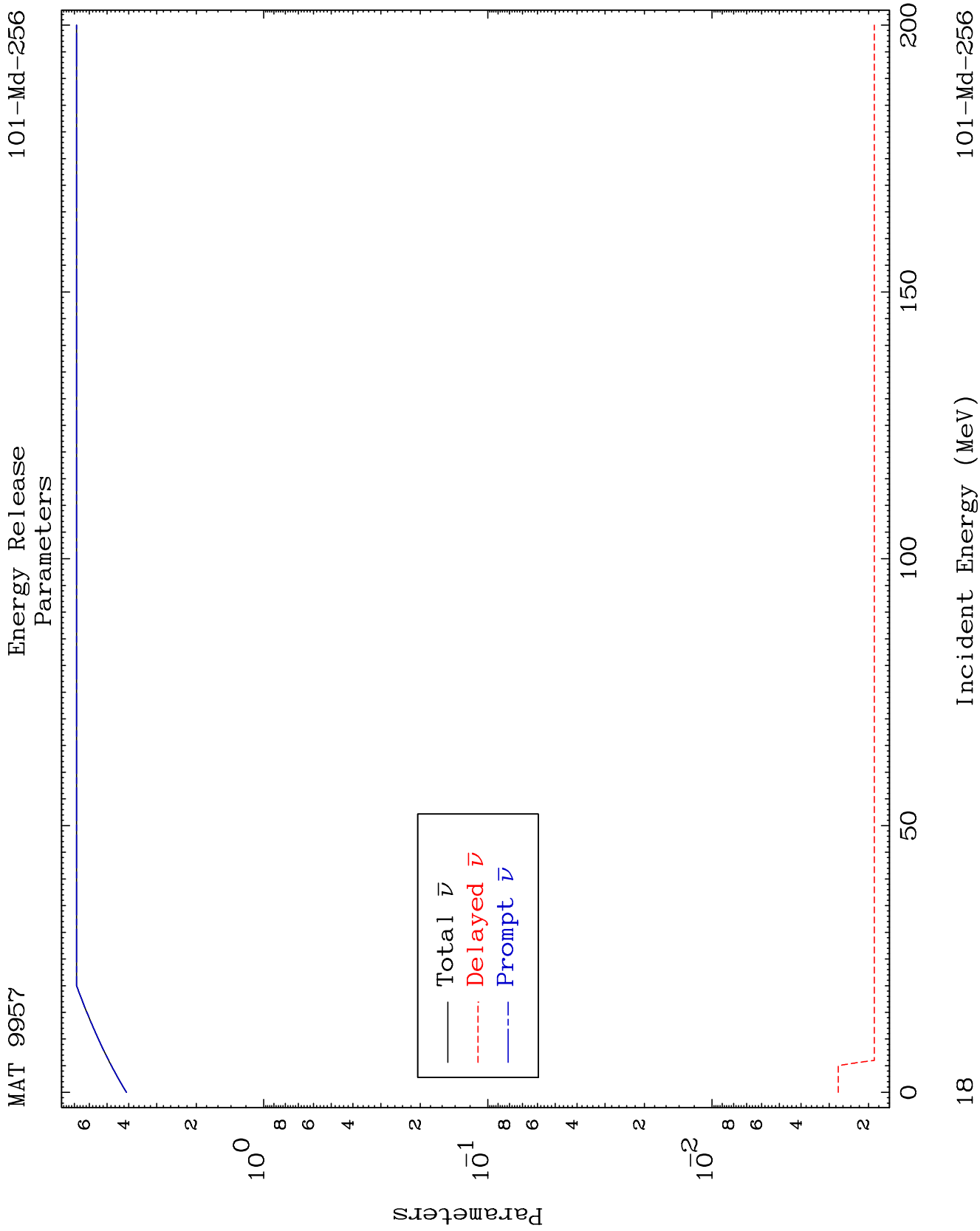
— (n, He-3)

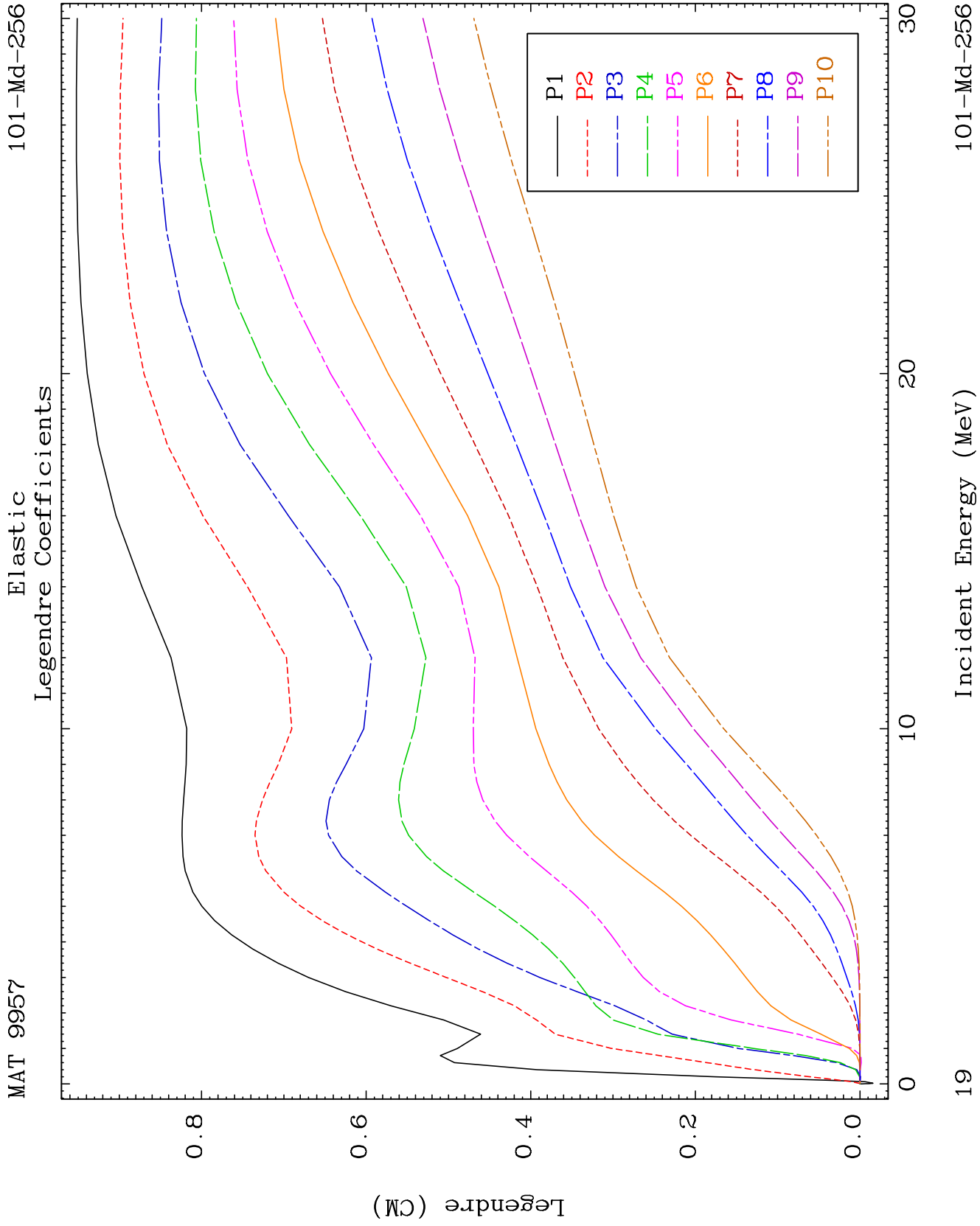
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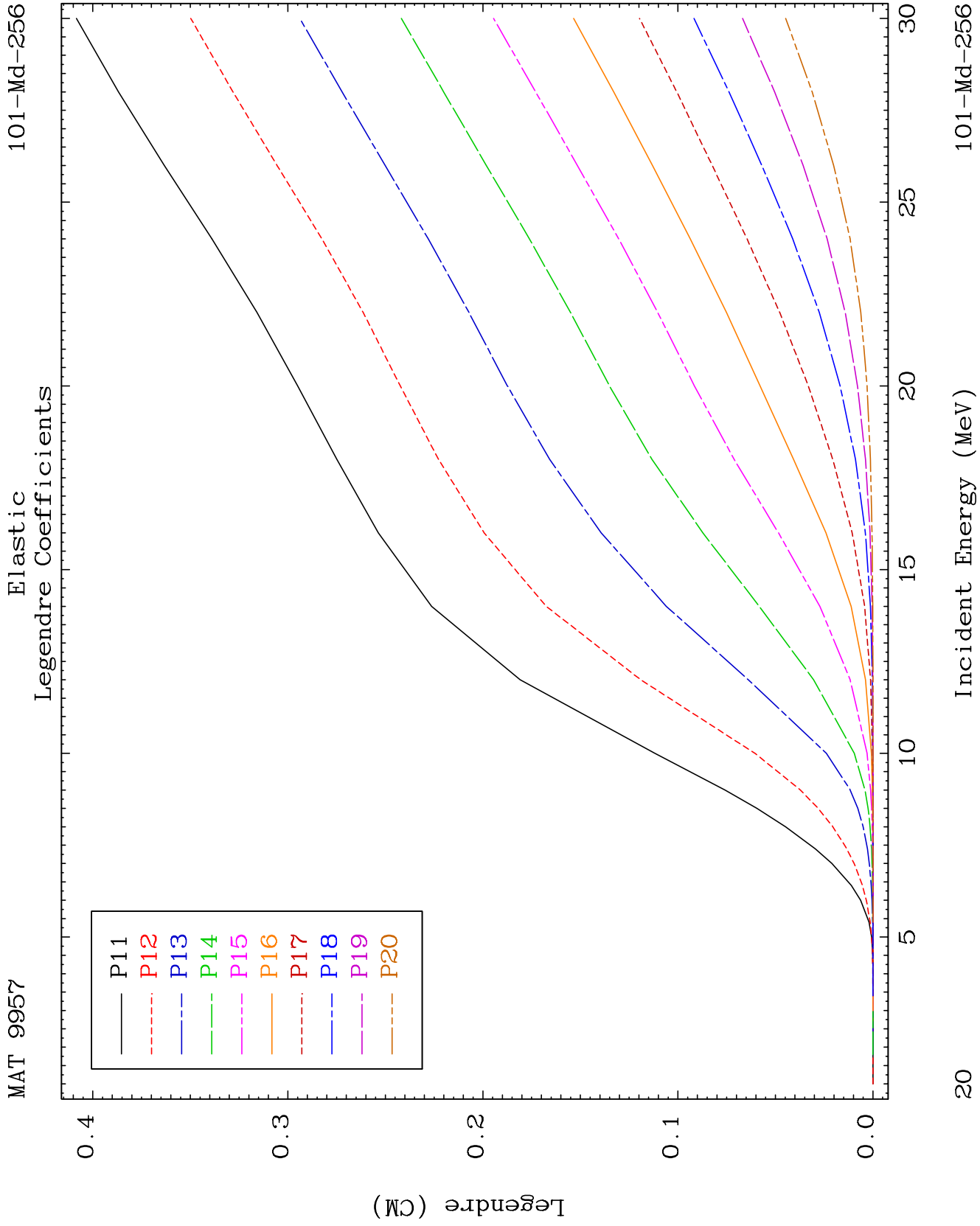
Incident Energy (MeV)

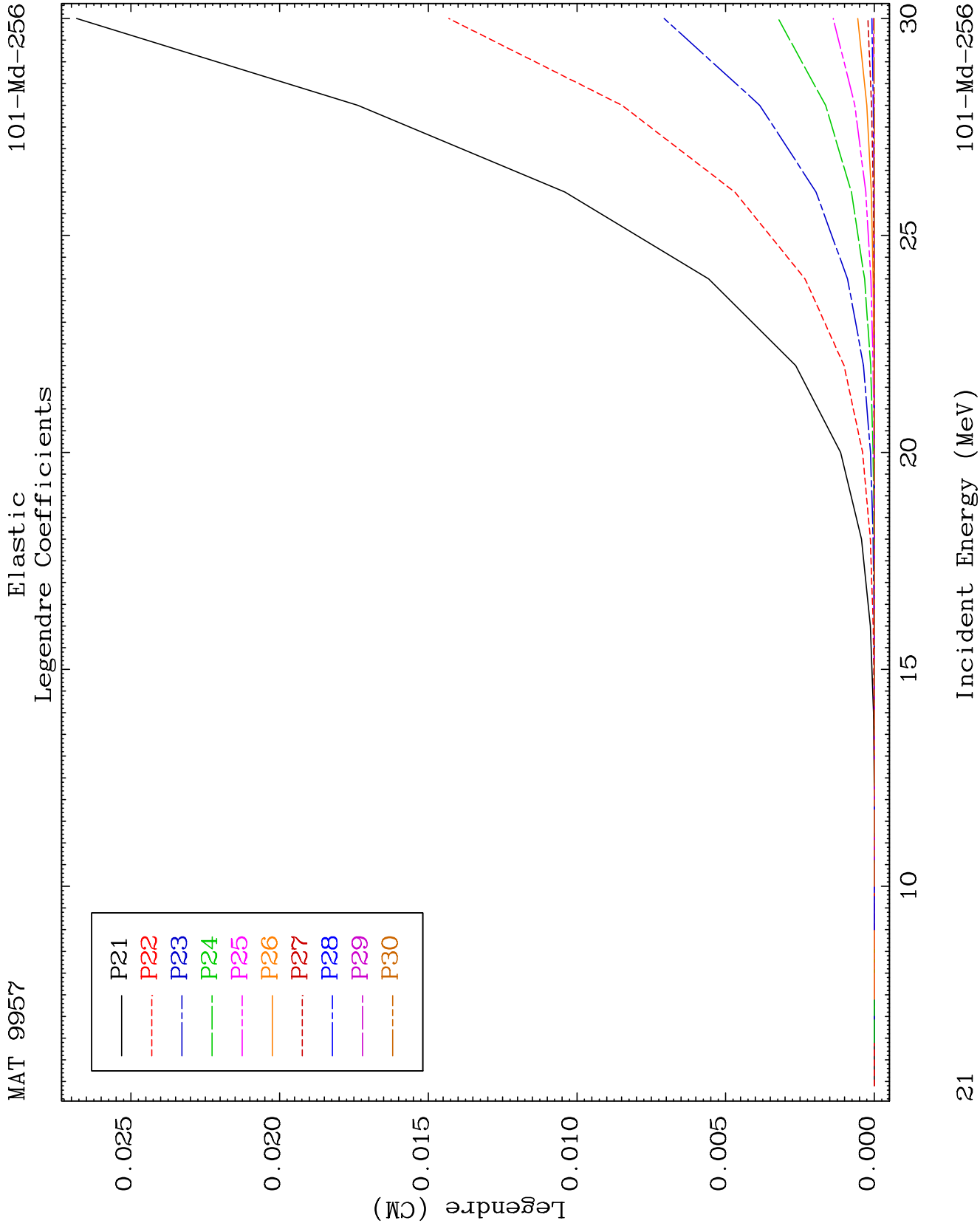
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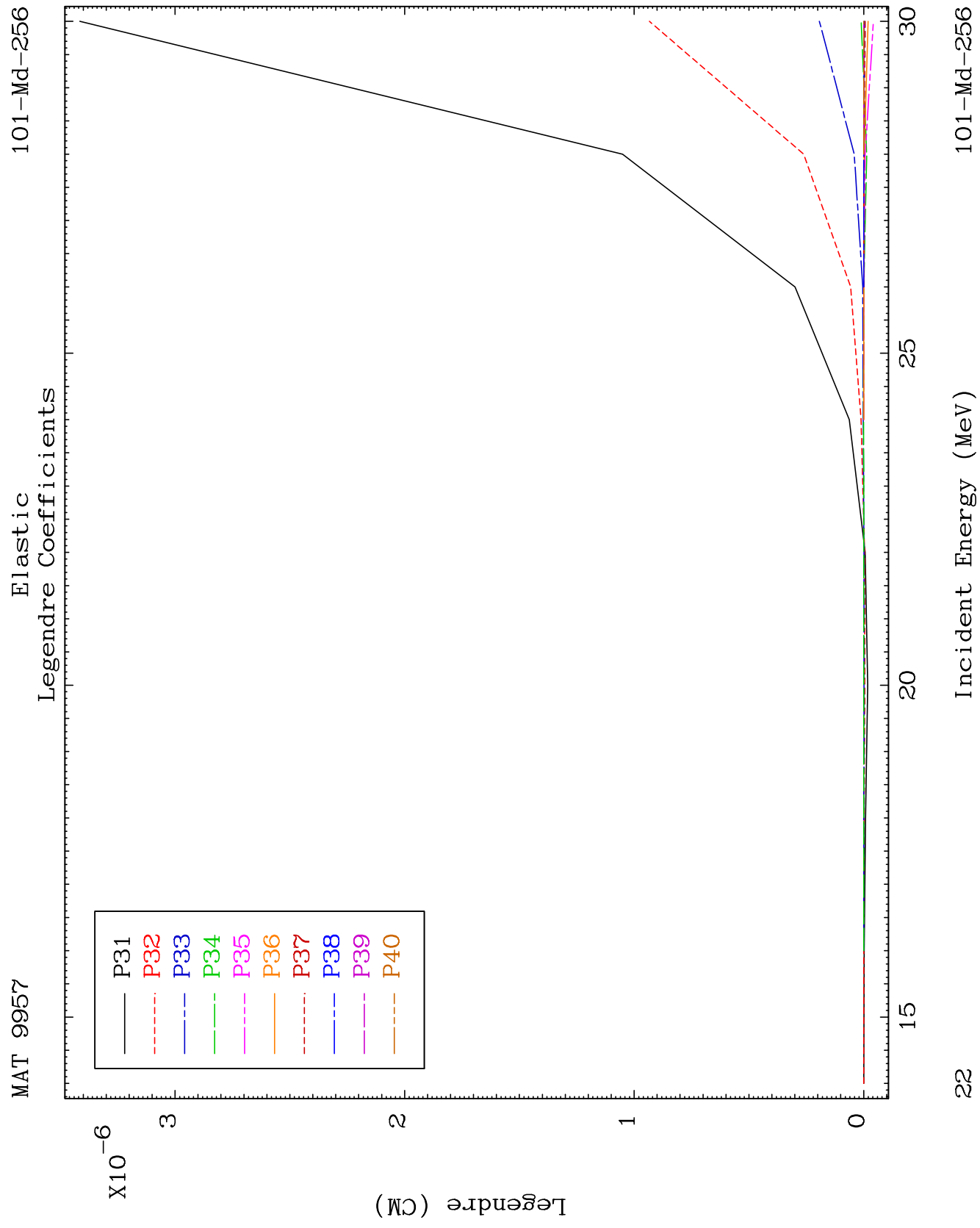








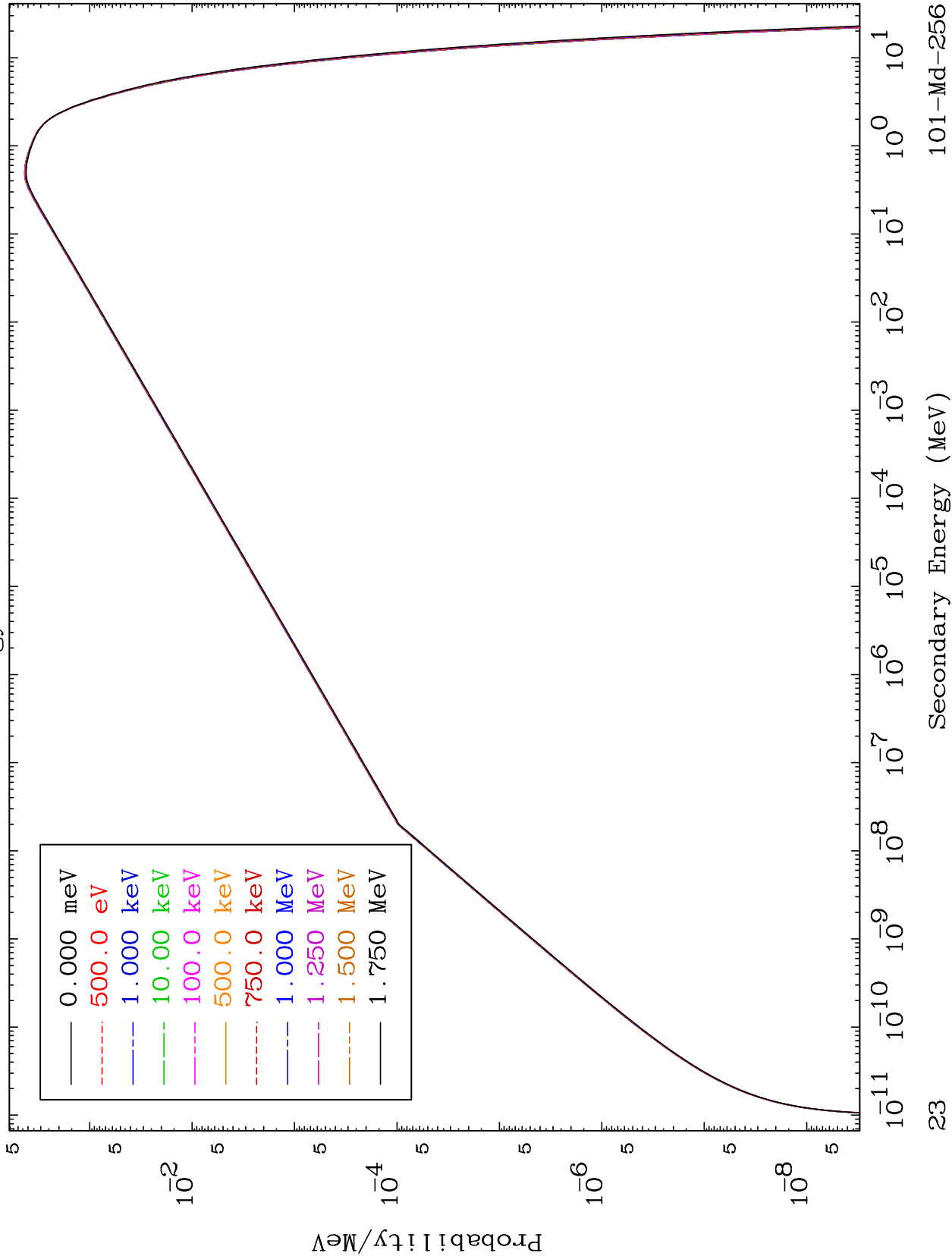




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Fission
Energy Distributions

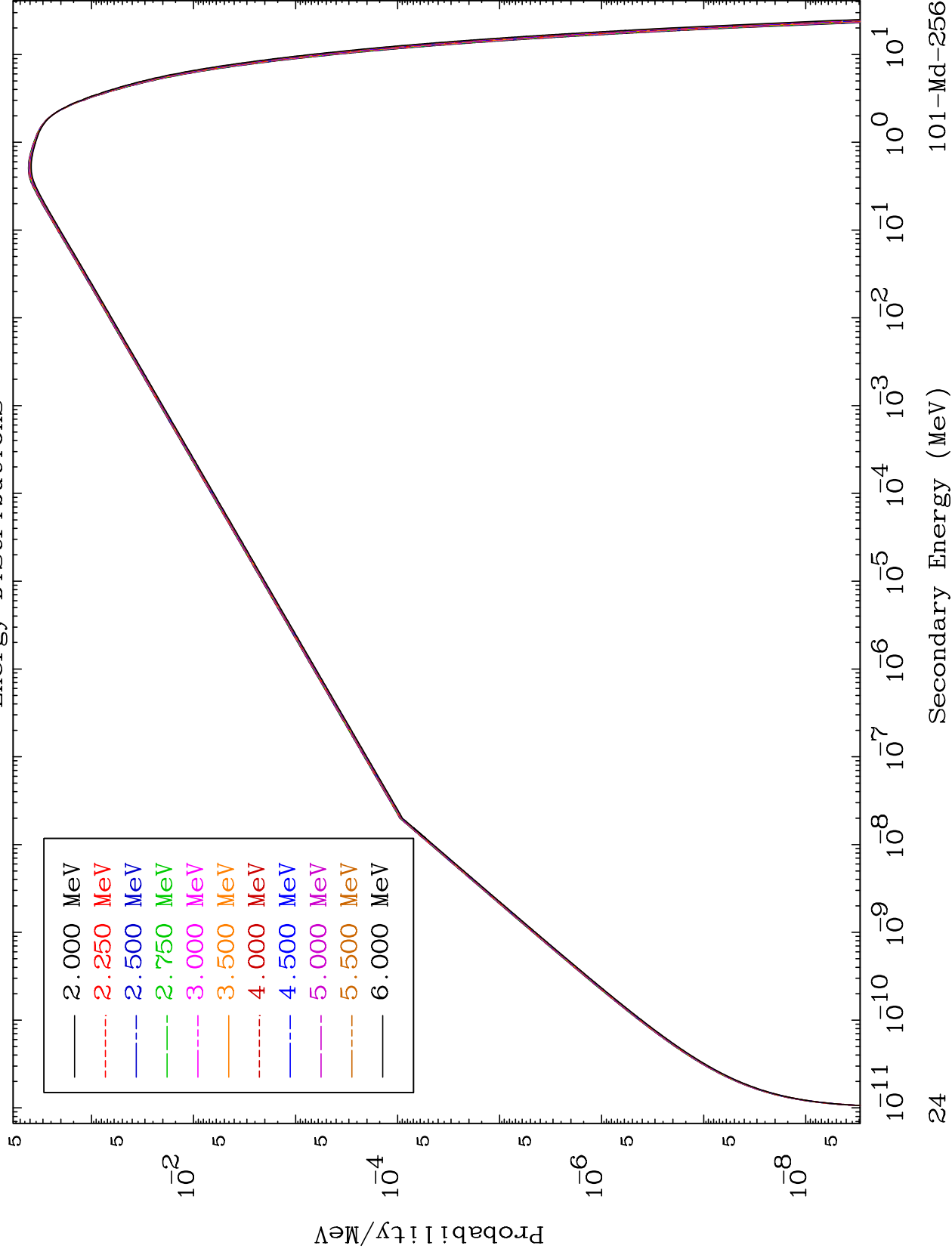
101-Md-256



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Fission
Energy Distributions

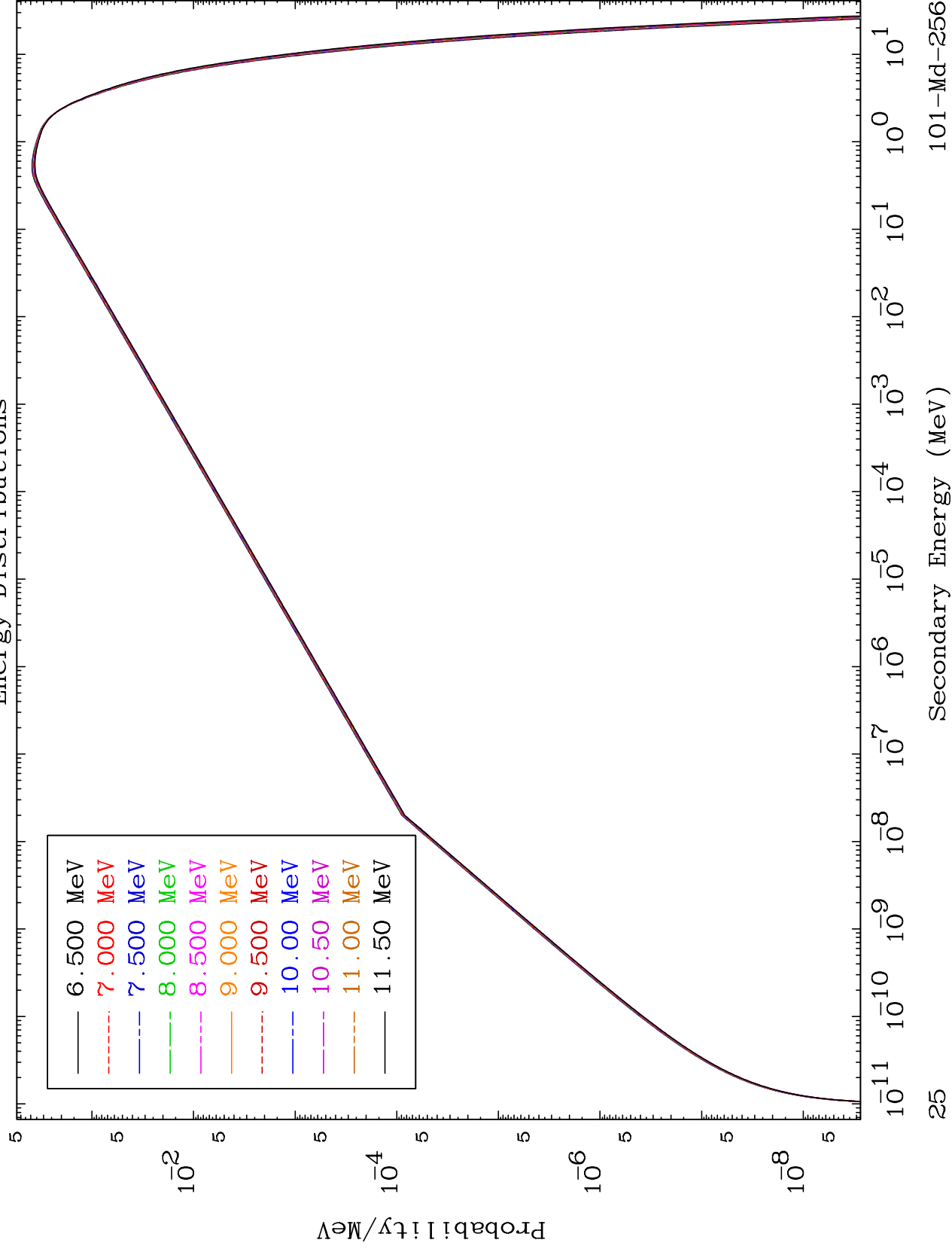
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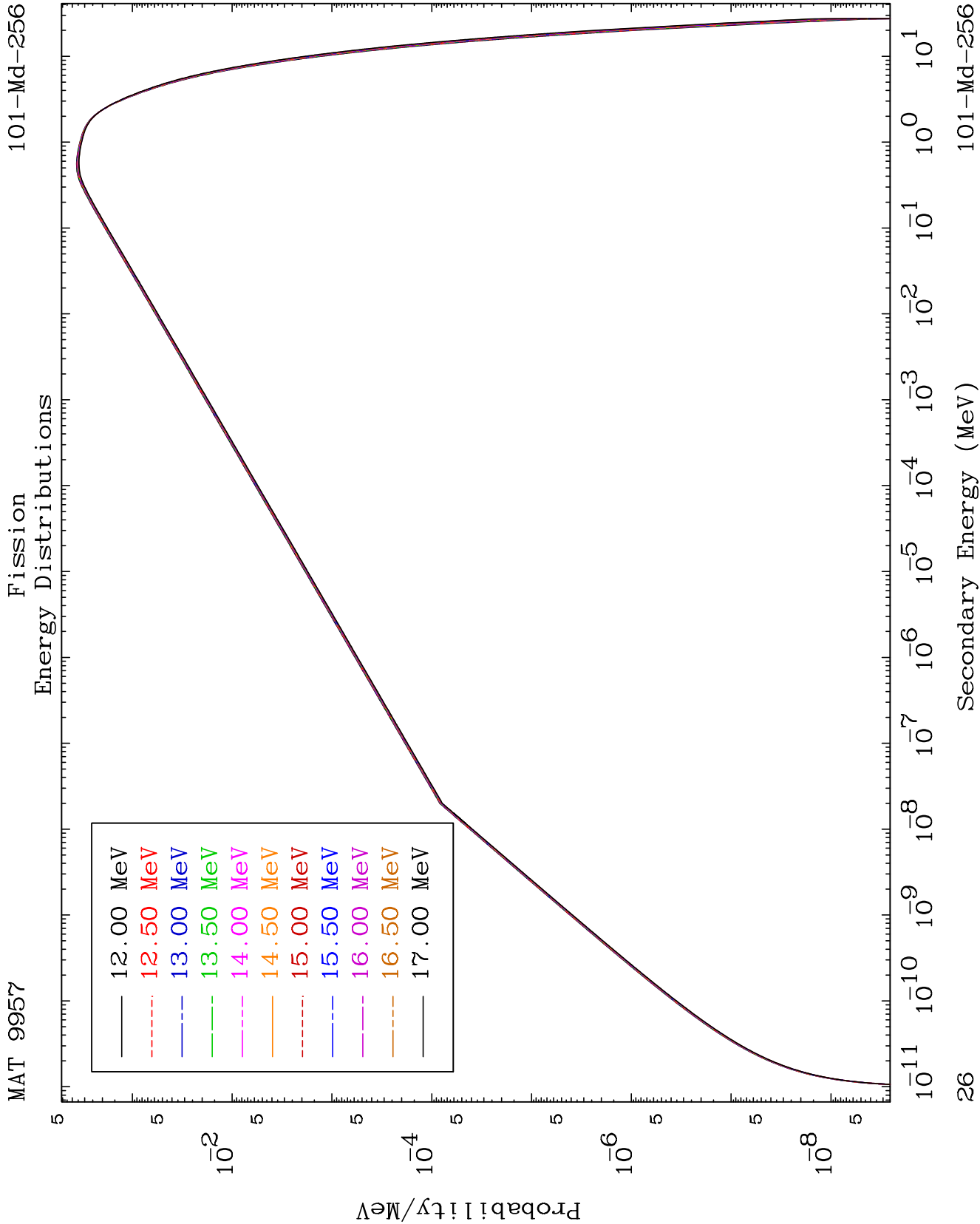


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Fission
Energy Distributions

101-Md-256

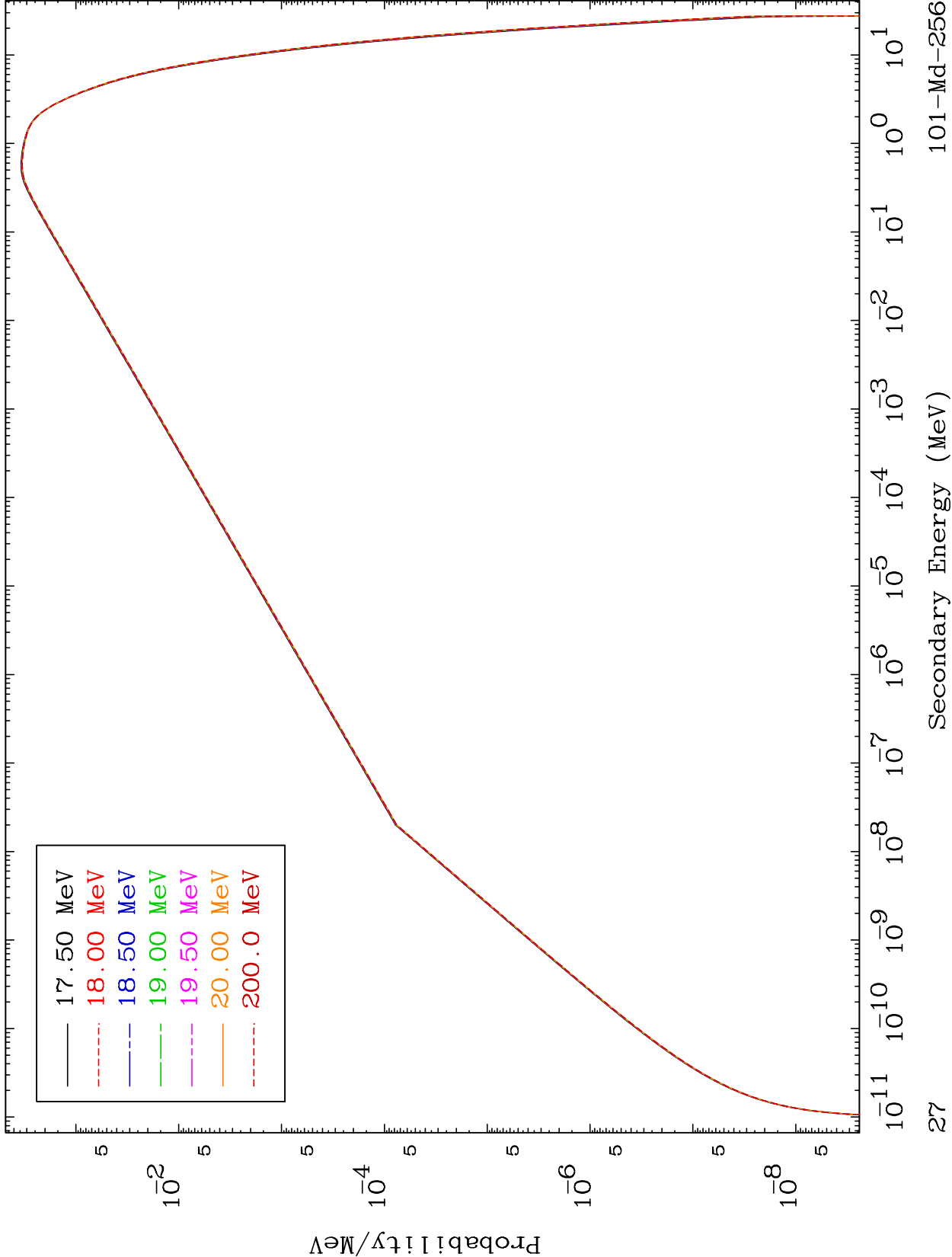




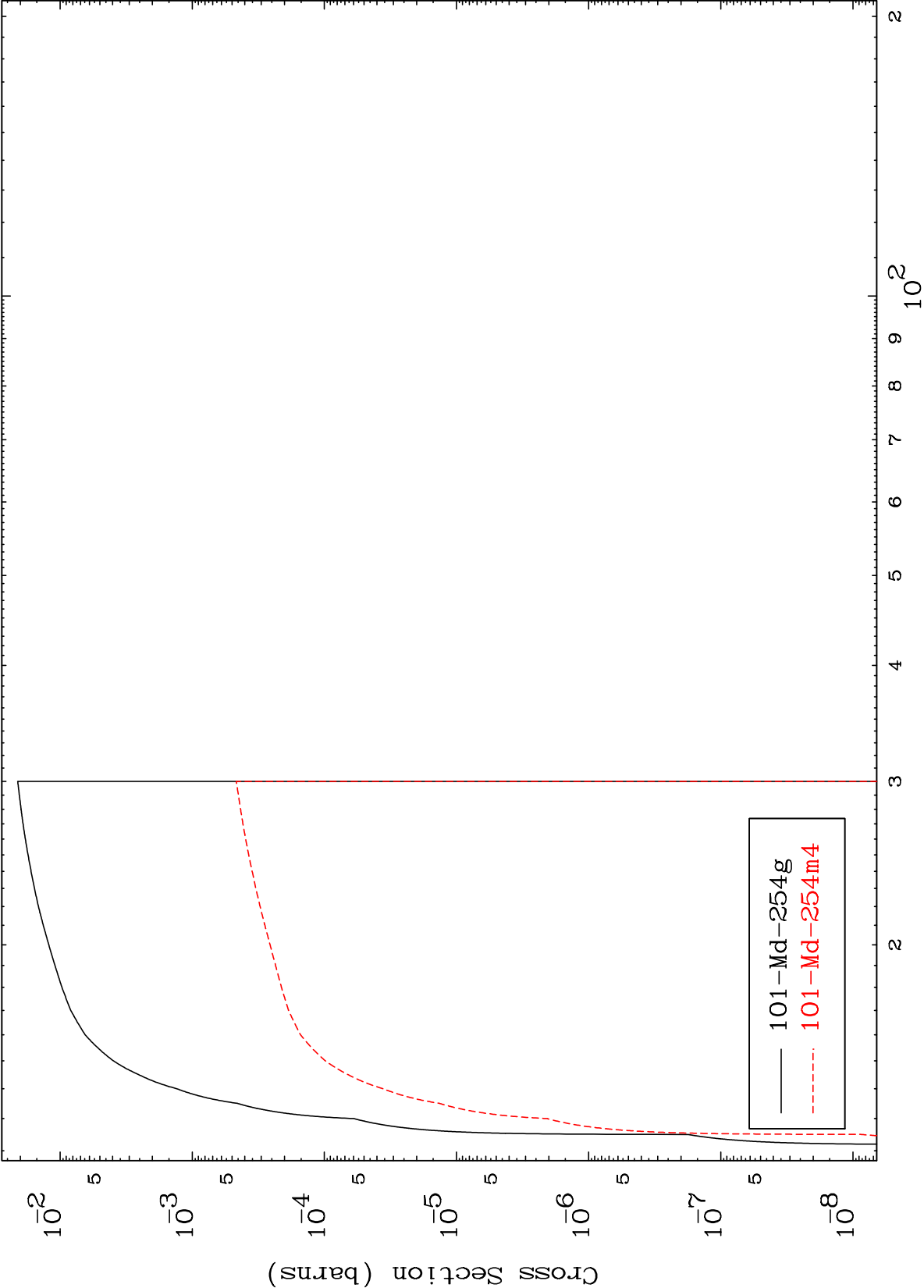
MAT 9957

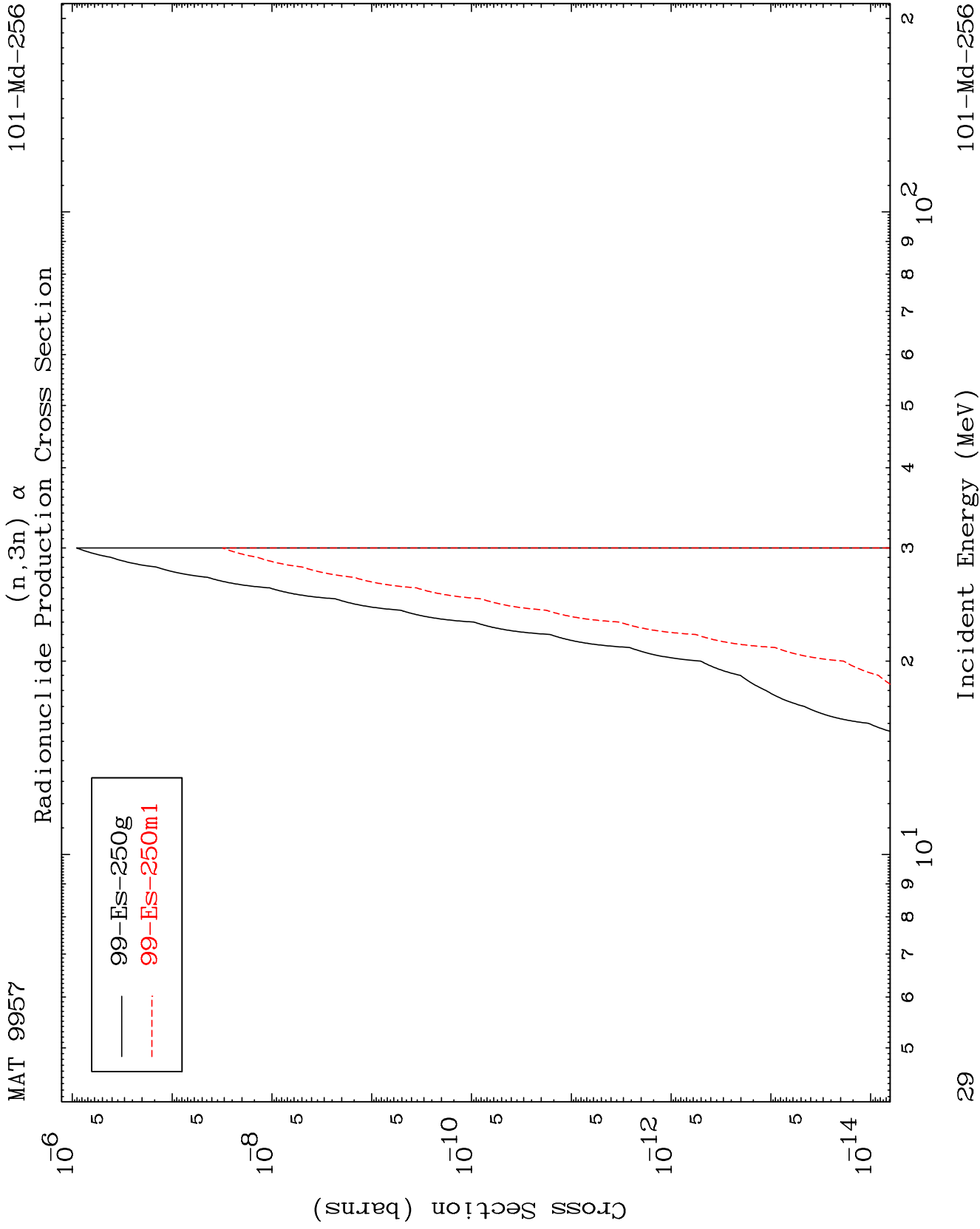
Fission
Energy Distributions

101-Md-256

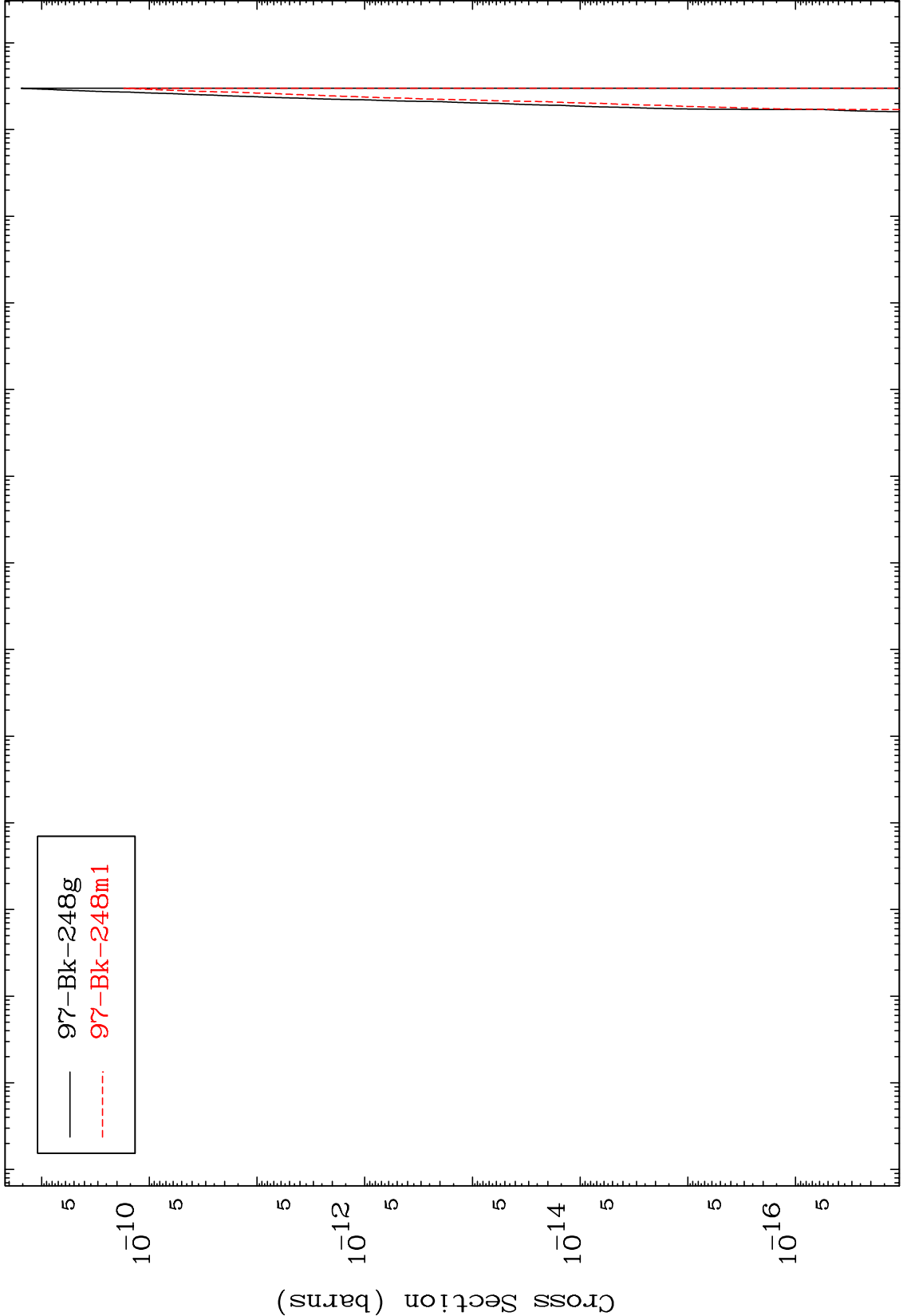


(n,3n)
Radionuclide Production Cross Section





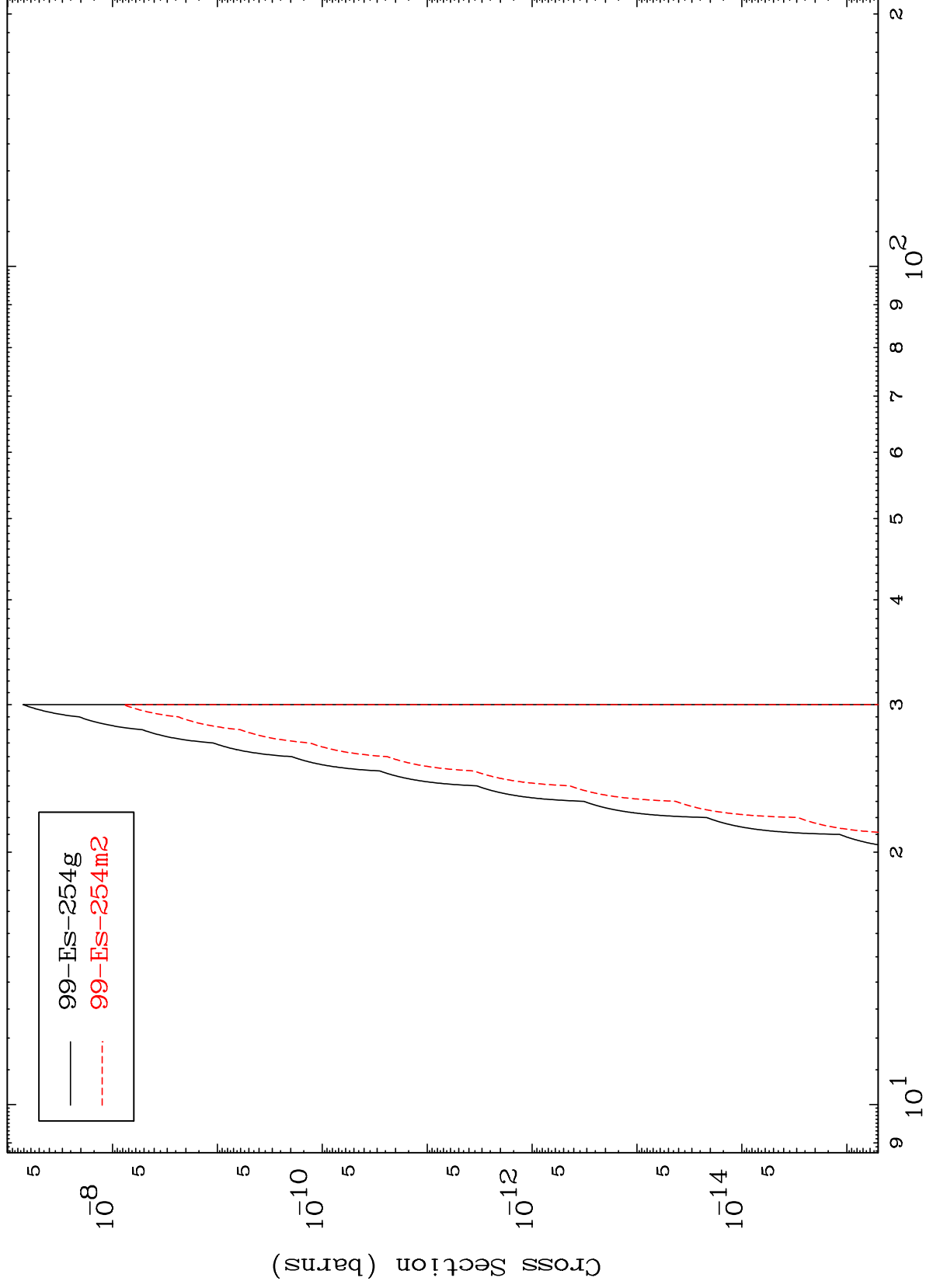
Radionuclide Production Cross Section



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(n,2n) p 101-Md-256

Radionuclide Production Cross Section



101-Md-256

Incident Energy (MeV)

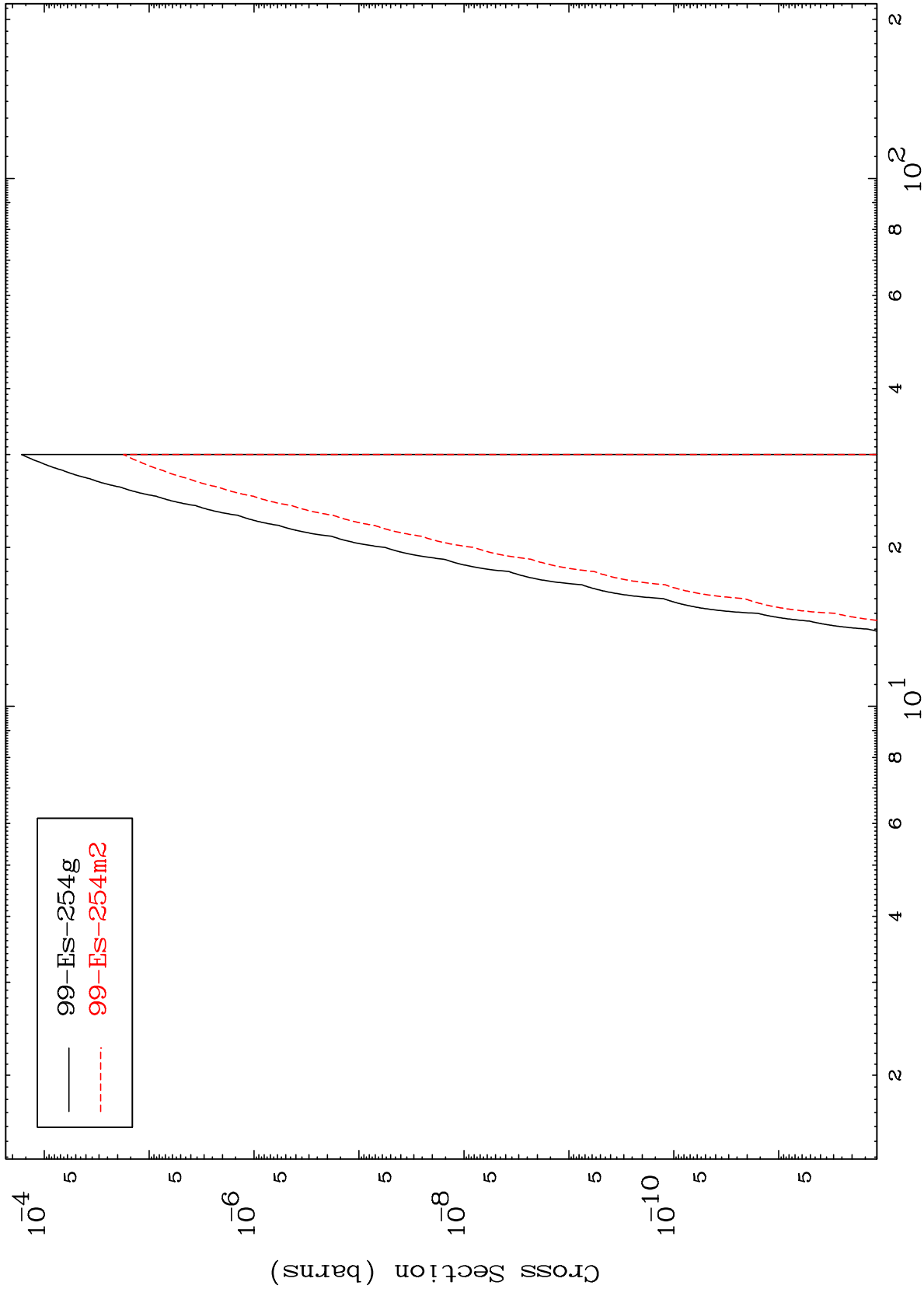
31

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(n,He-3)

101-Md-256

Radionuclide Production Cross Section



32

Incident Energy (MeV)

101-Md-256