

Program EVALPLOT
(Version 2021-1)

by

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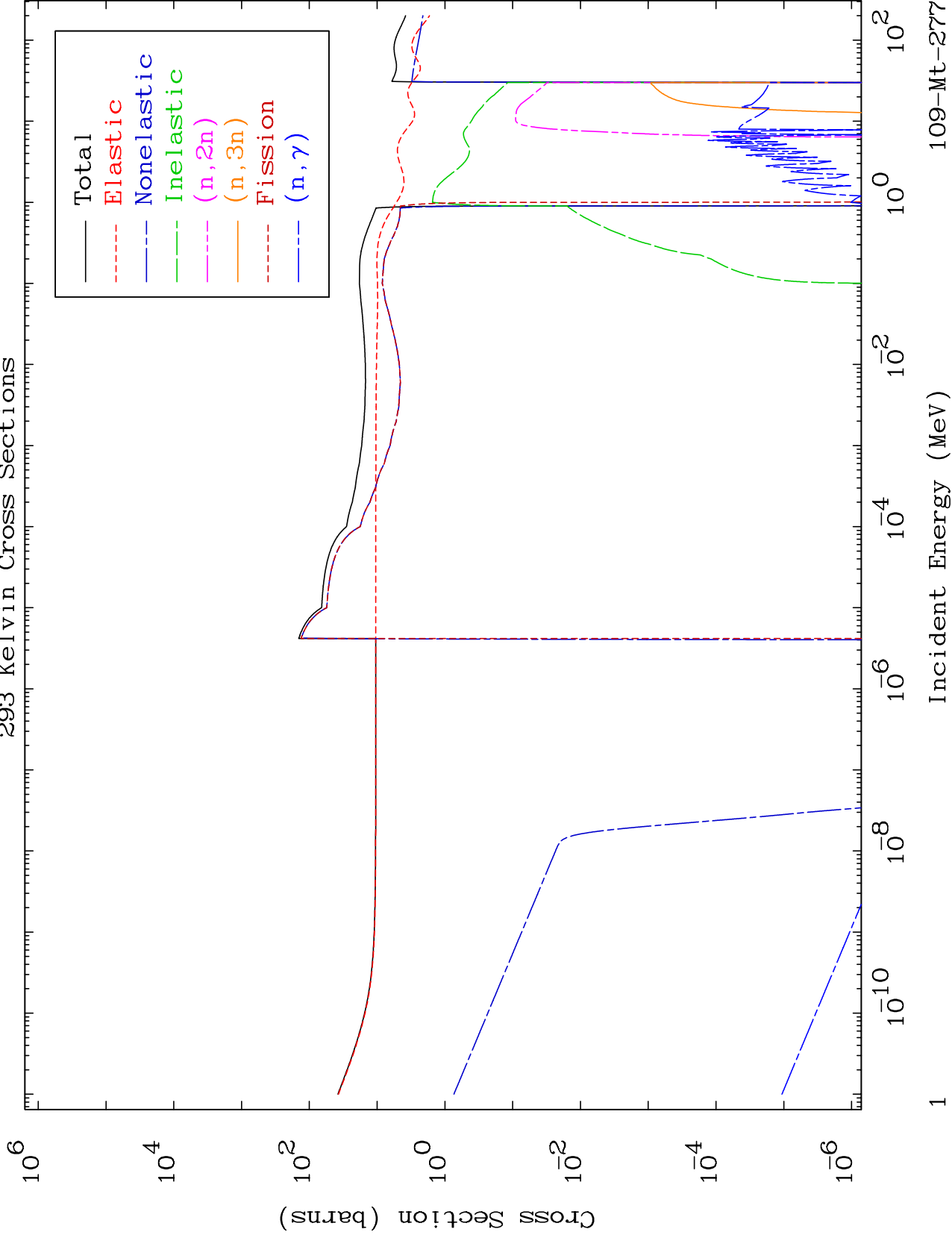
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 977

Neutron Major
293 Kelvin Cross Sections

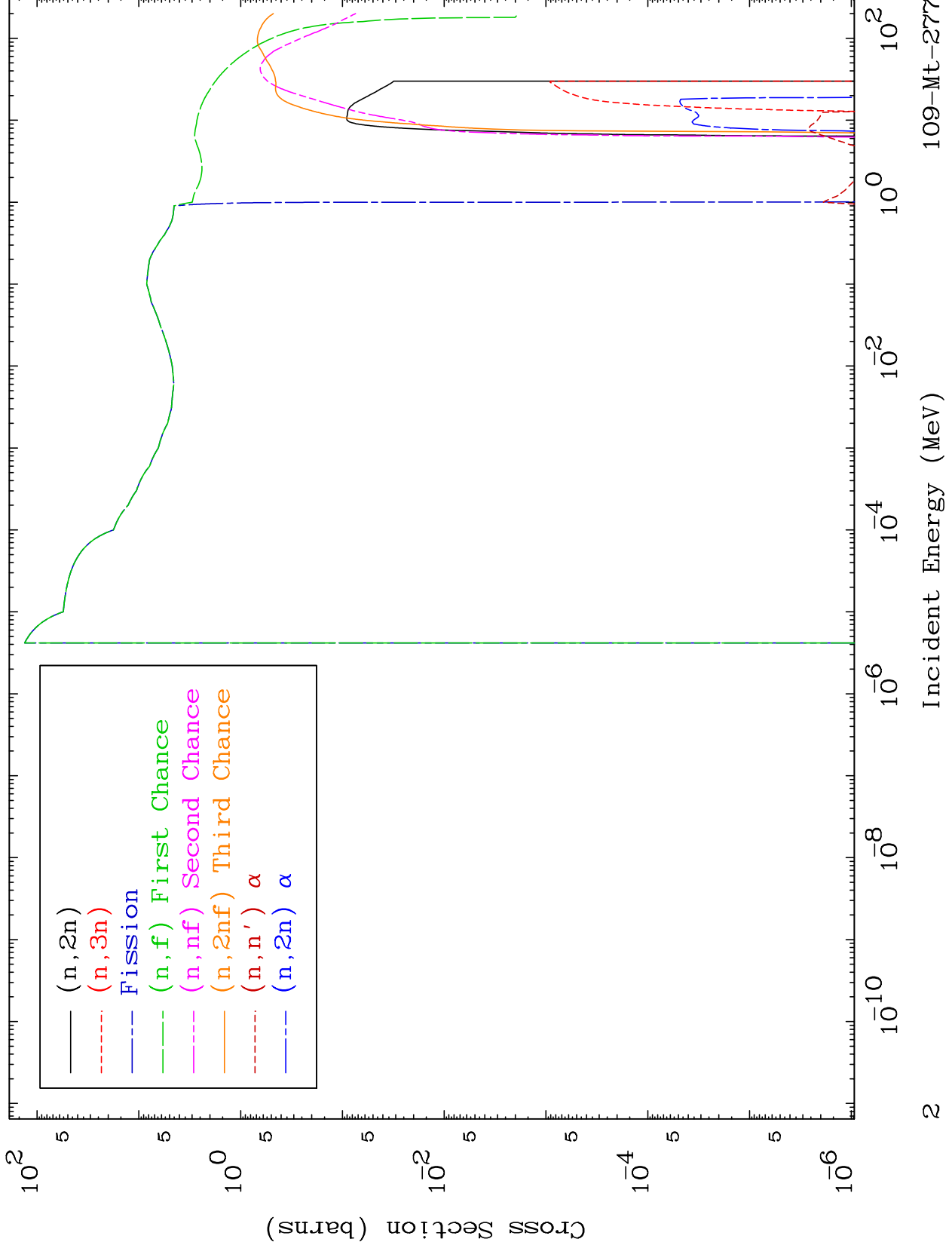
109-Mt-277

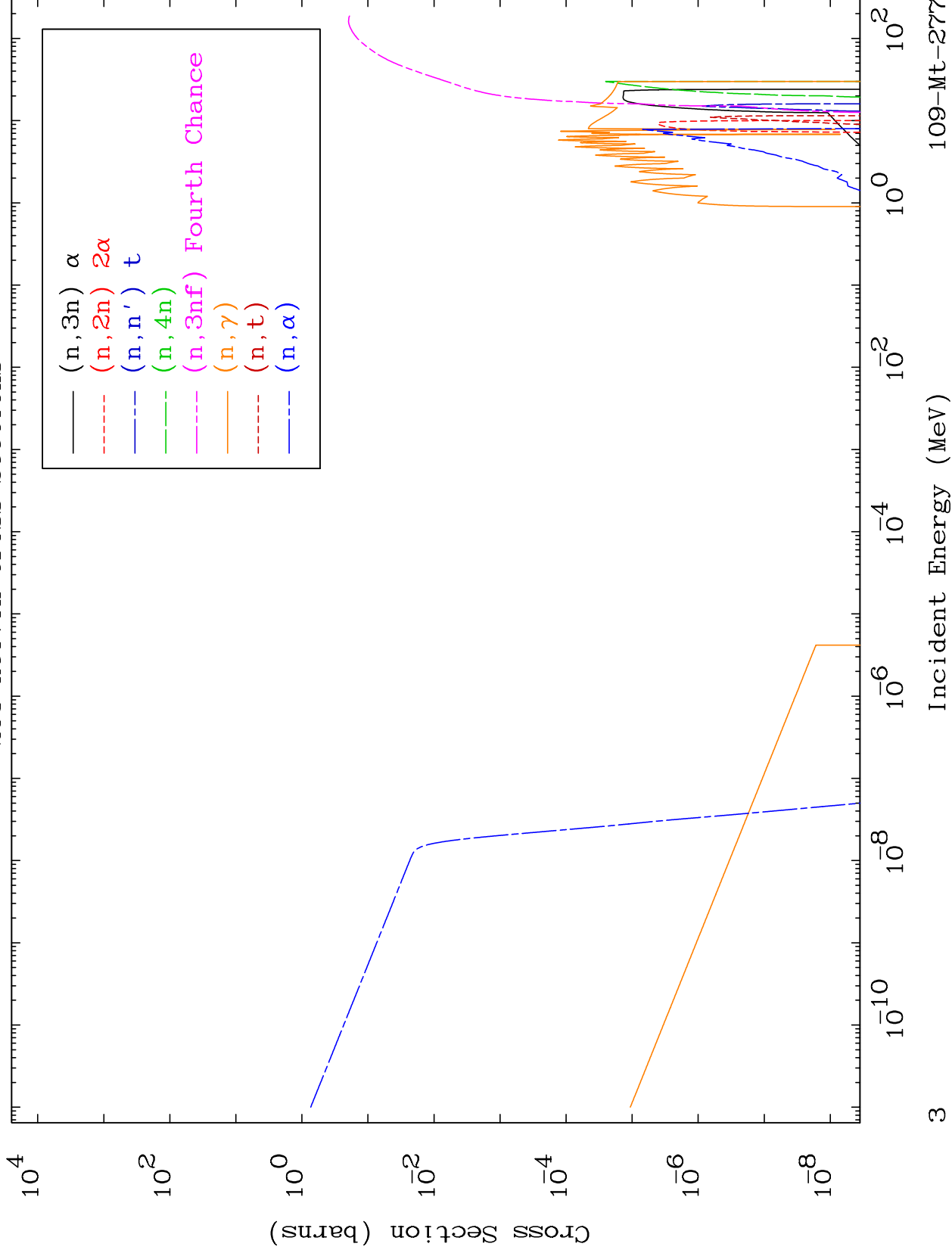


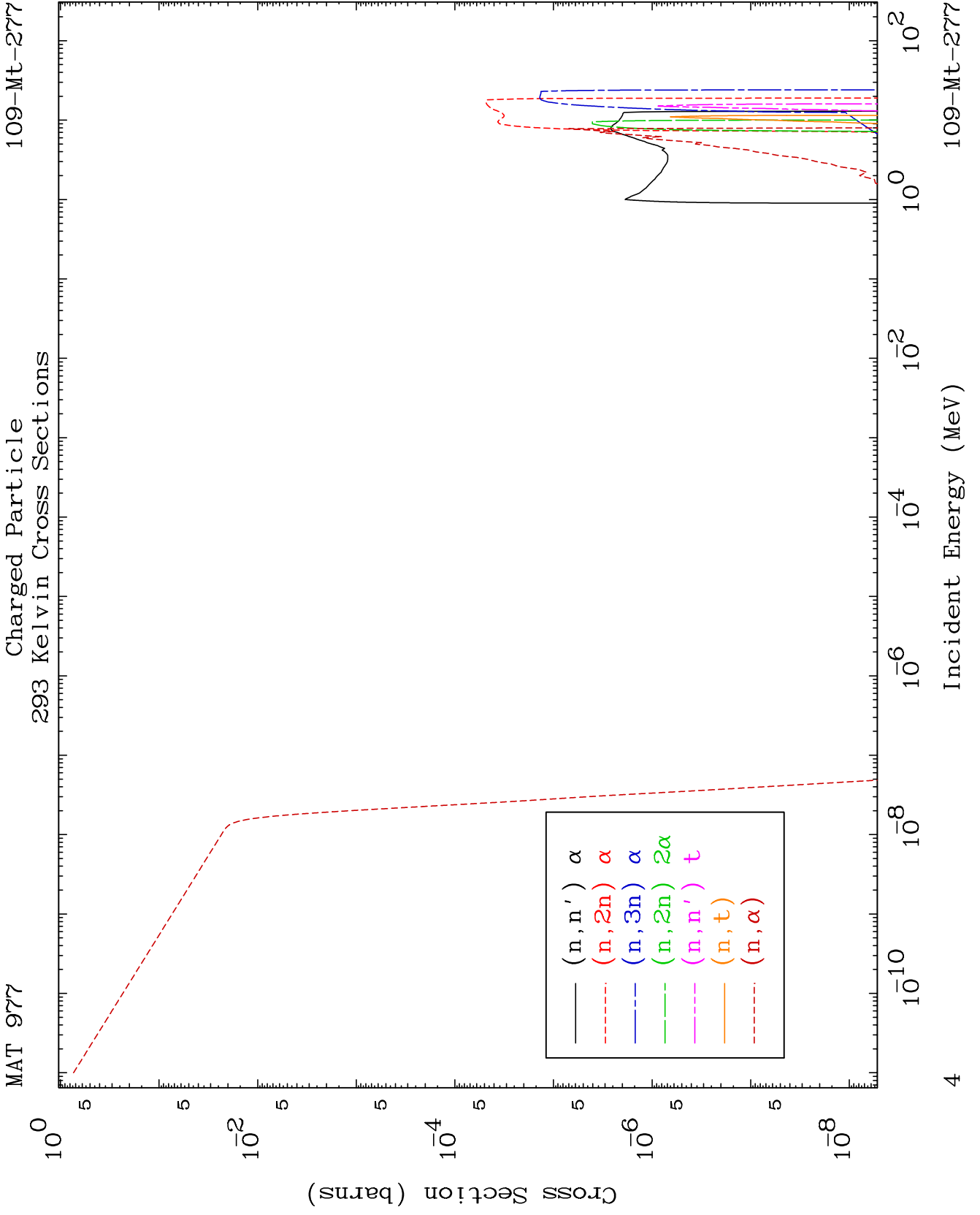
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Neutron Absorption
293 Kelvin Cross Sections

109-Mt-277



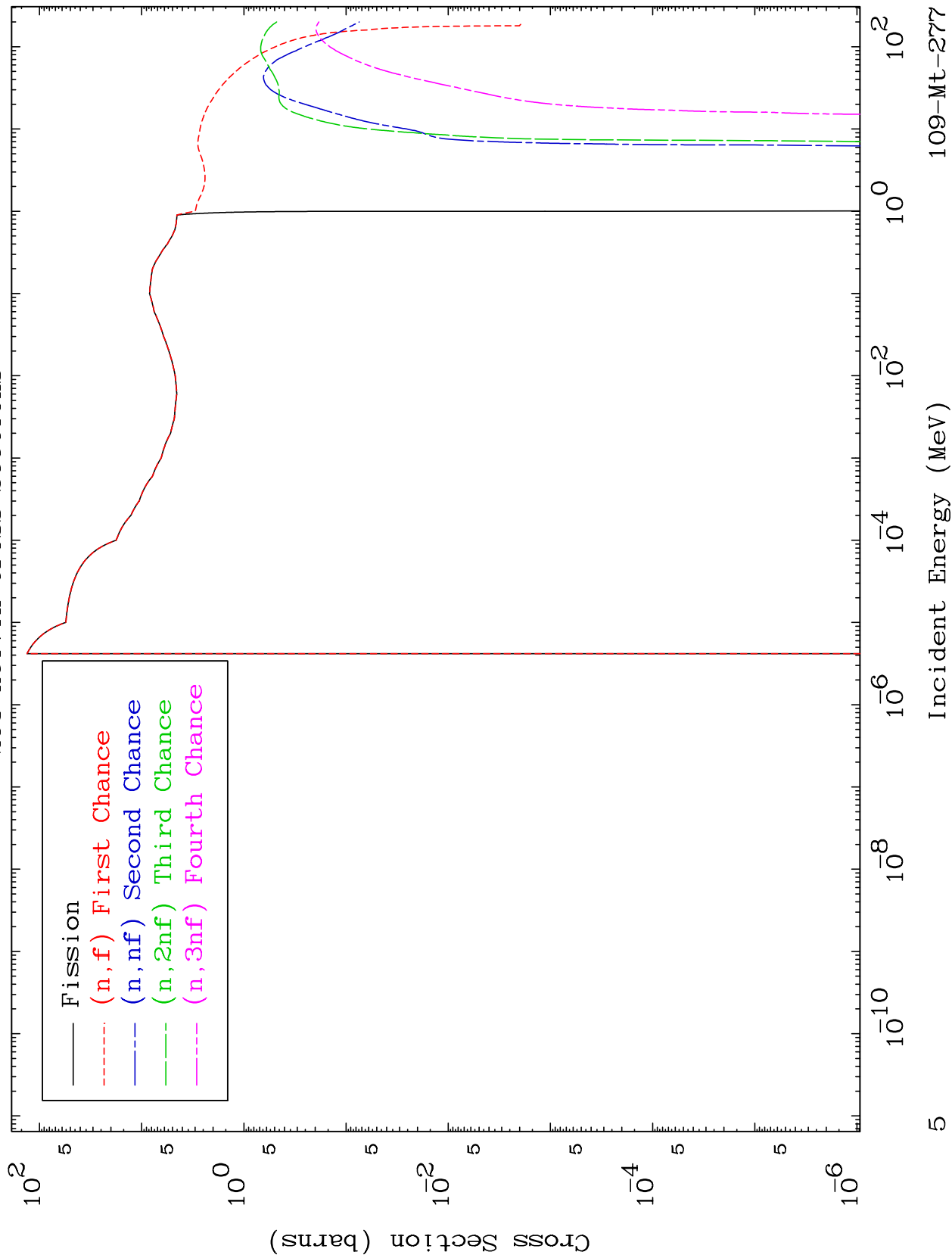




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Fission
293 Kelvin Cross Sections

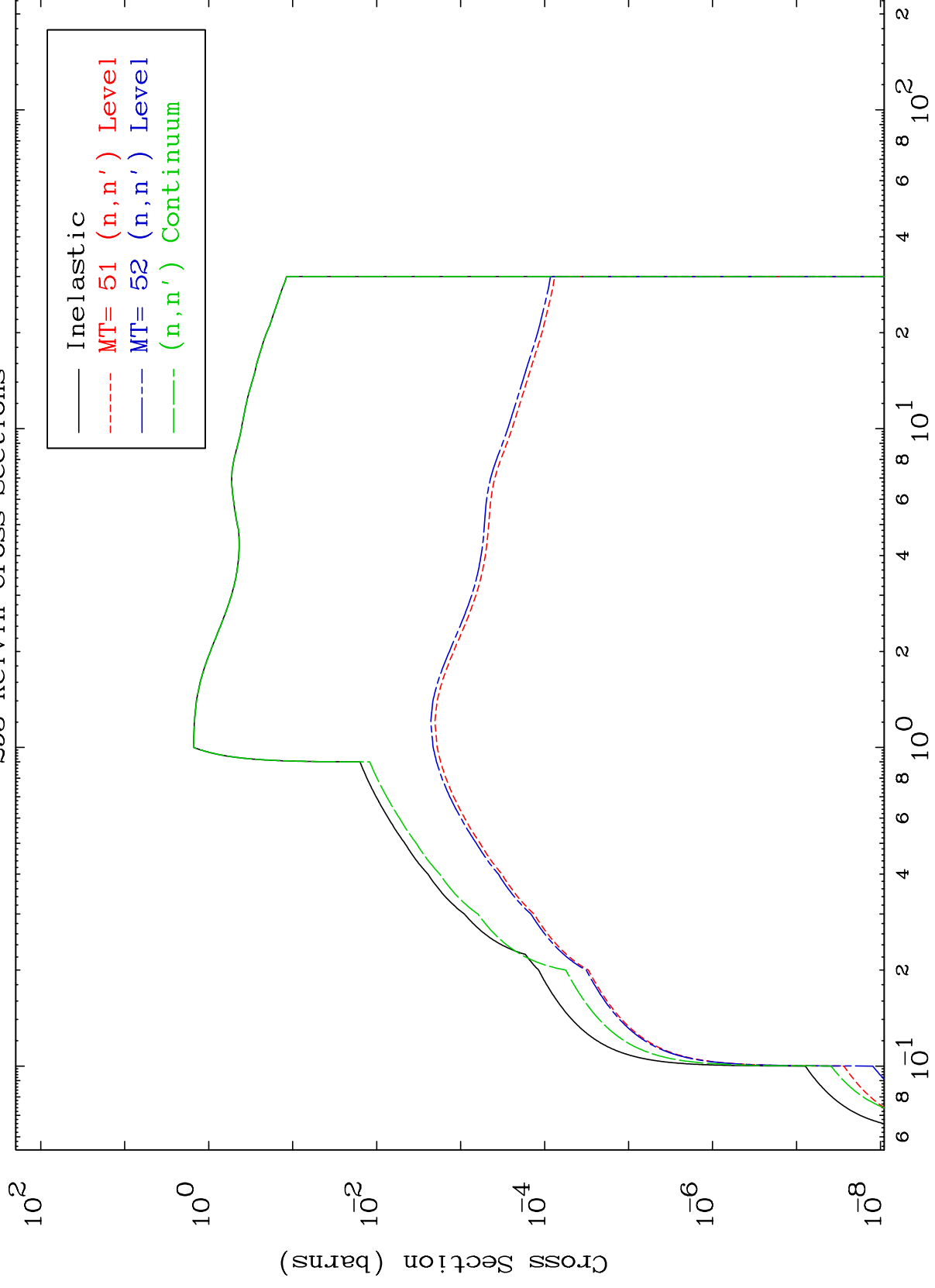
109-Mt-277



MAT 977

(n,n') Levels
293 Kelvin Cross Sections

109-Mt-277



6

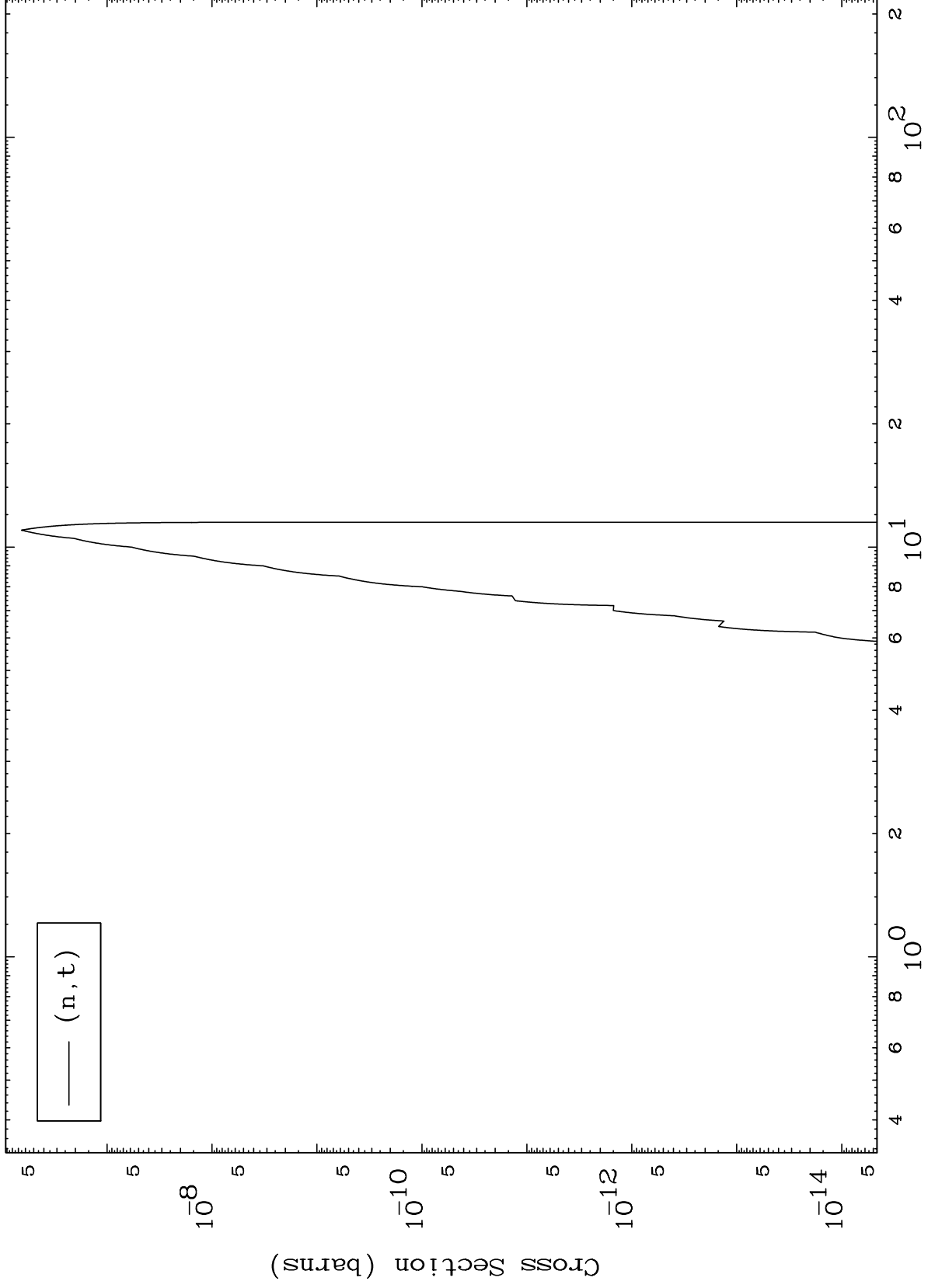
Incident Energy (MeV)

109-Mt-277

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(n,t) Levels
293 Kelvin Cross Sections

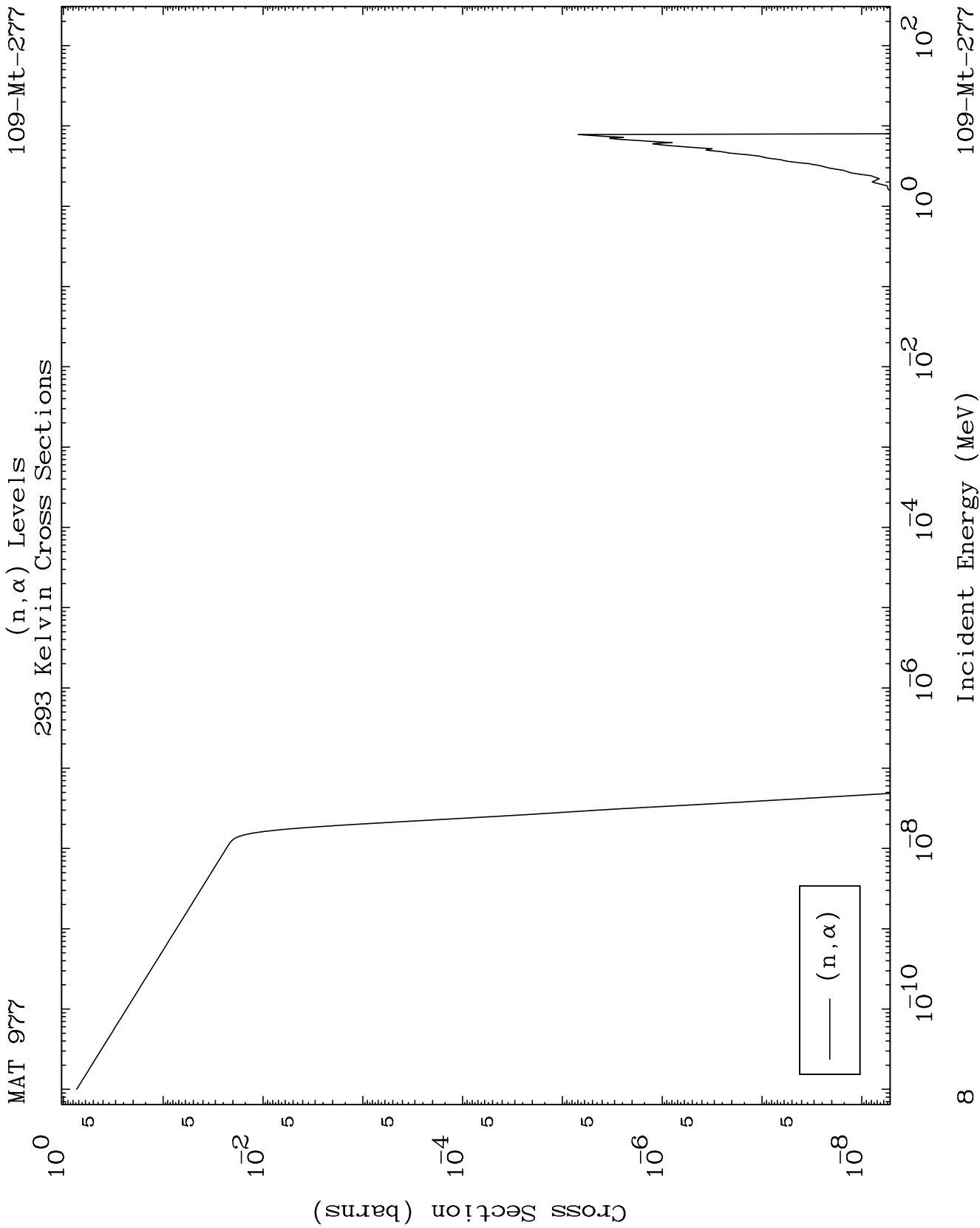
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7

Incident Energy (MeV)

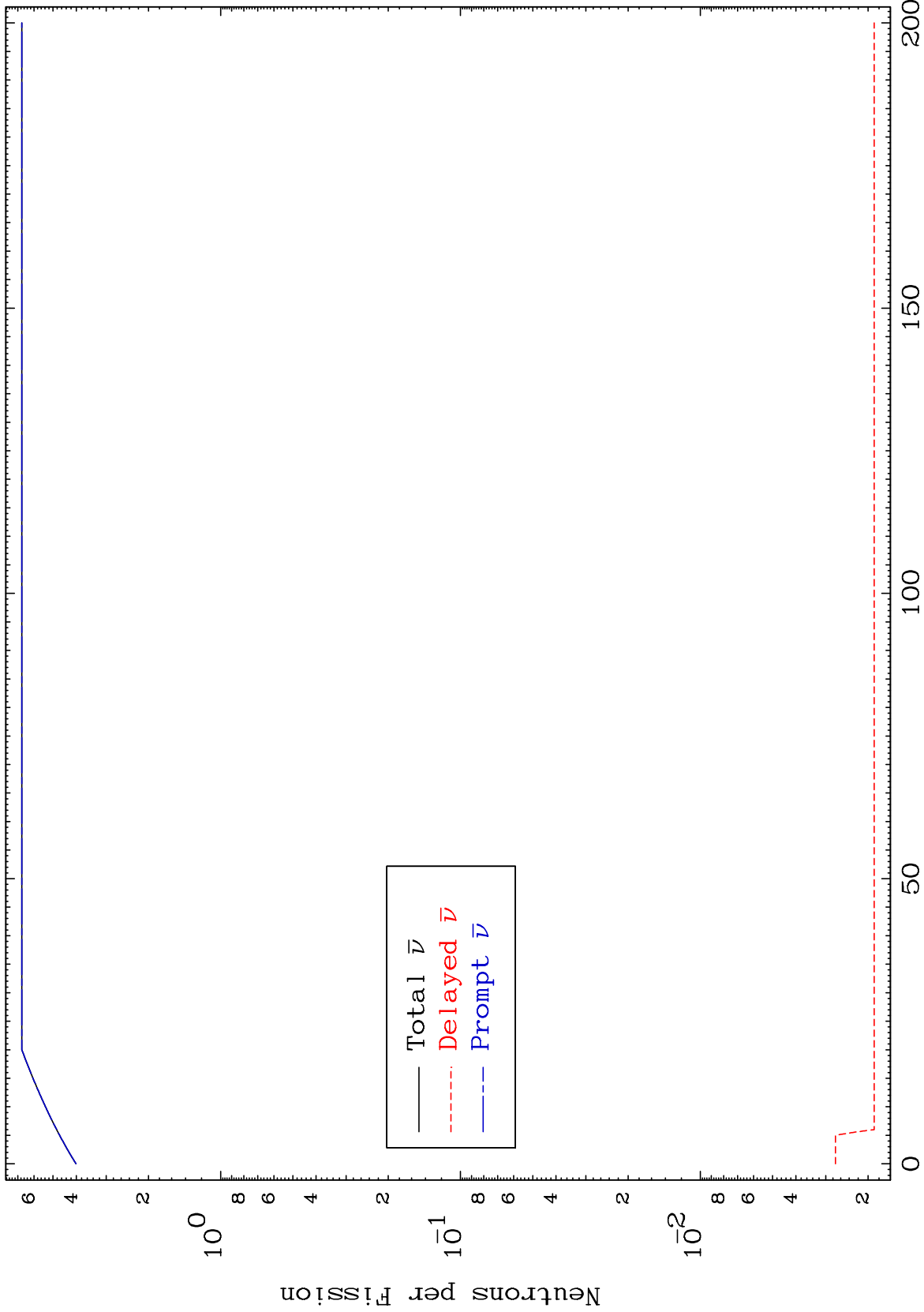
109-Mt-277



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109-Mt-277

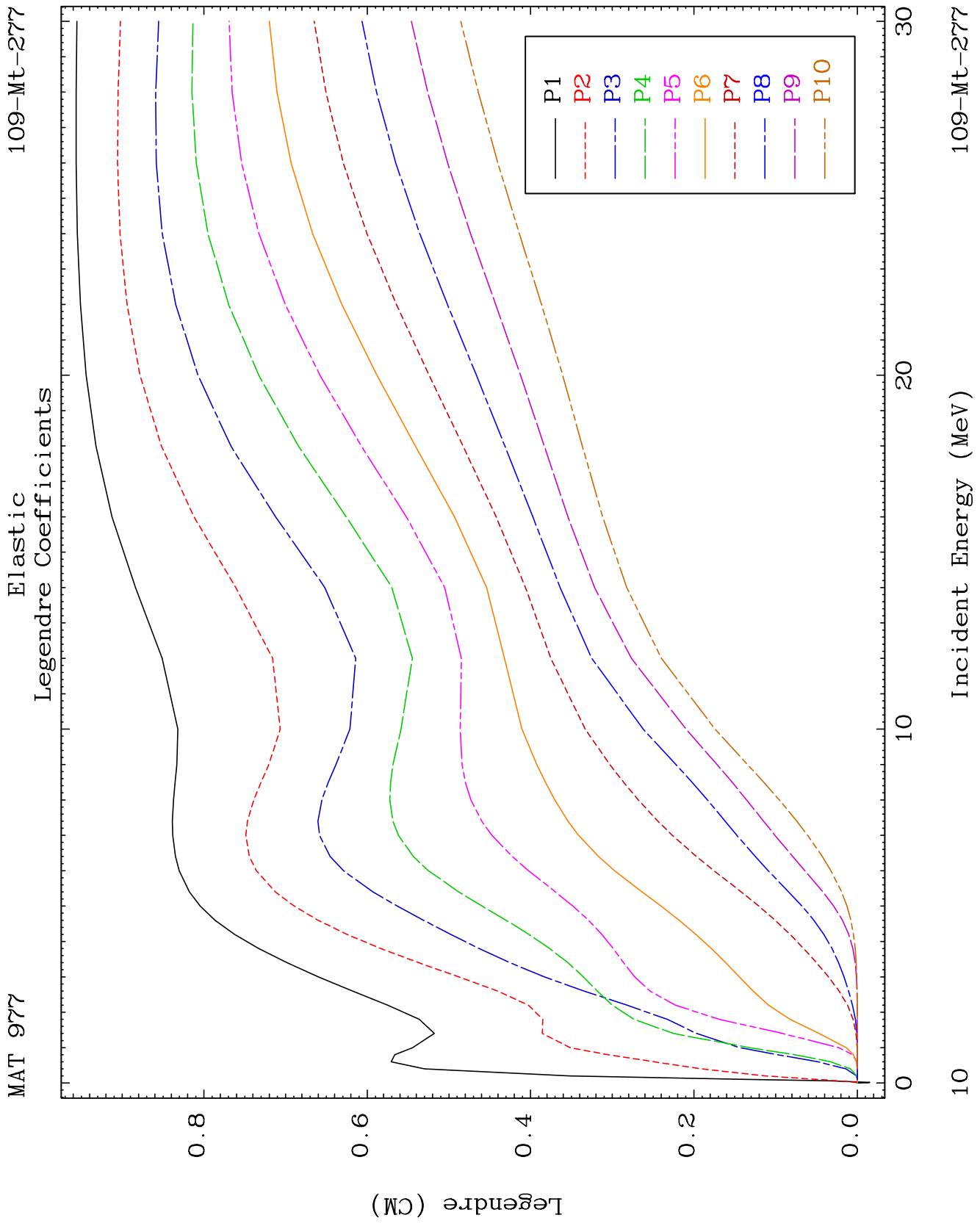
Neutrons
per Fission

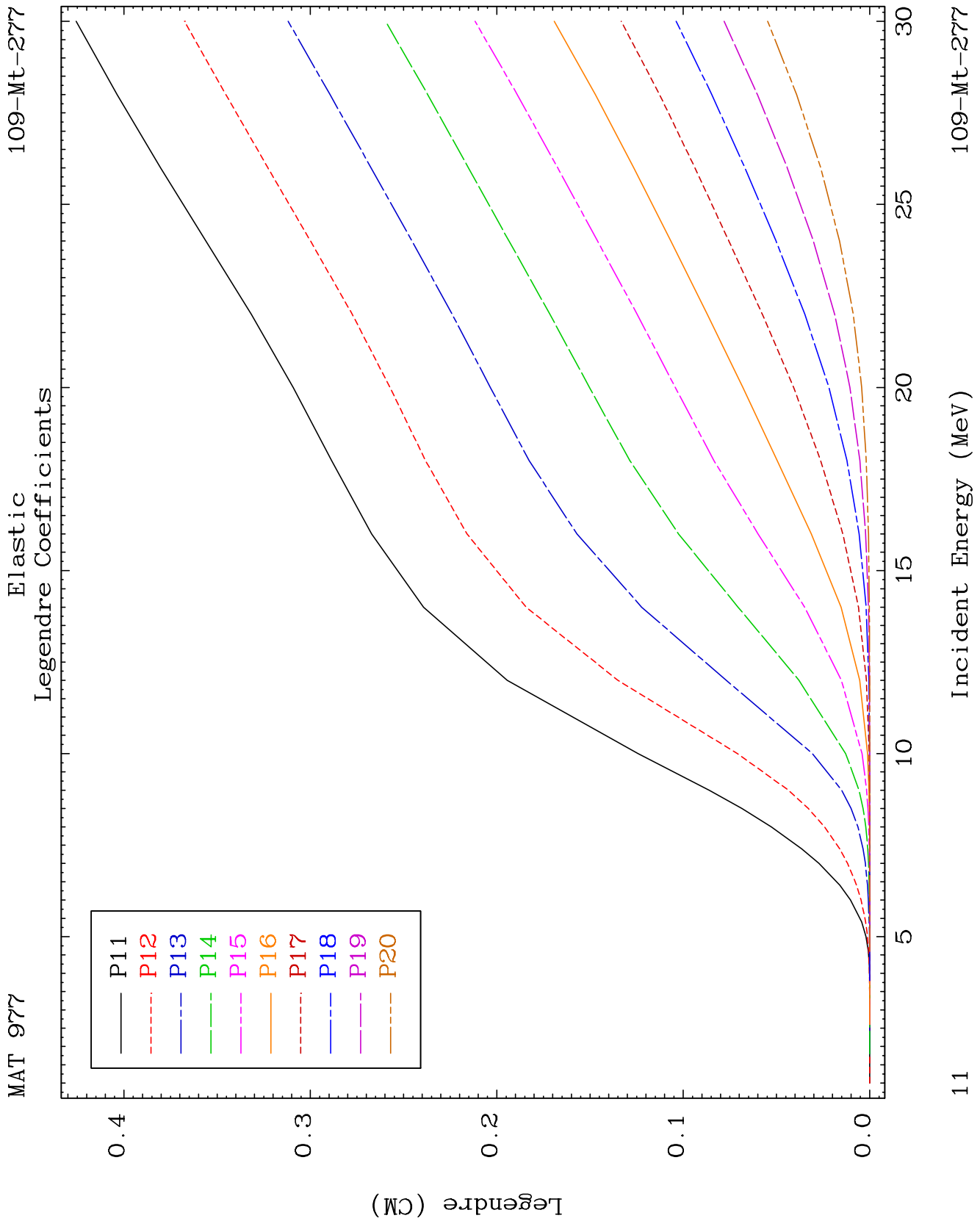


9

Incident Energy (MeV)

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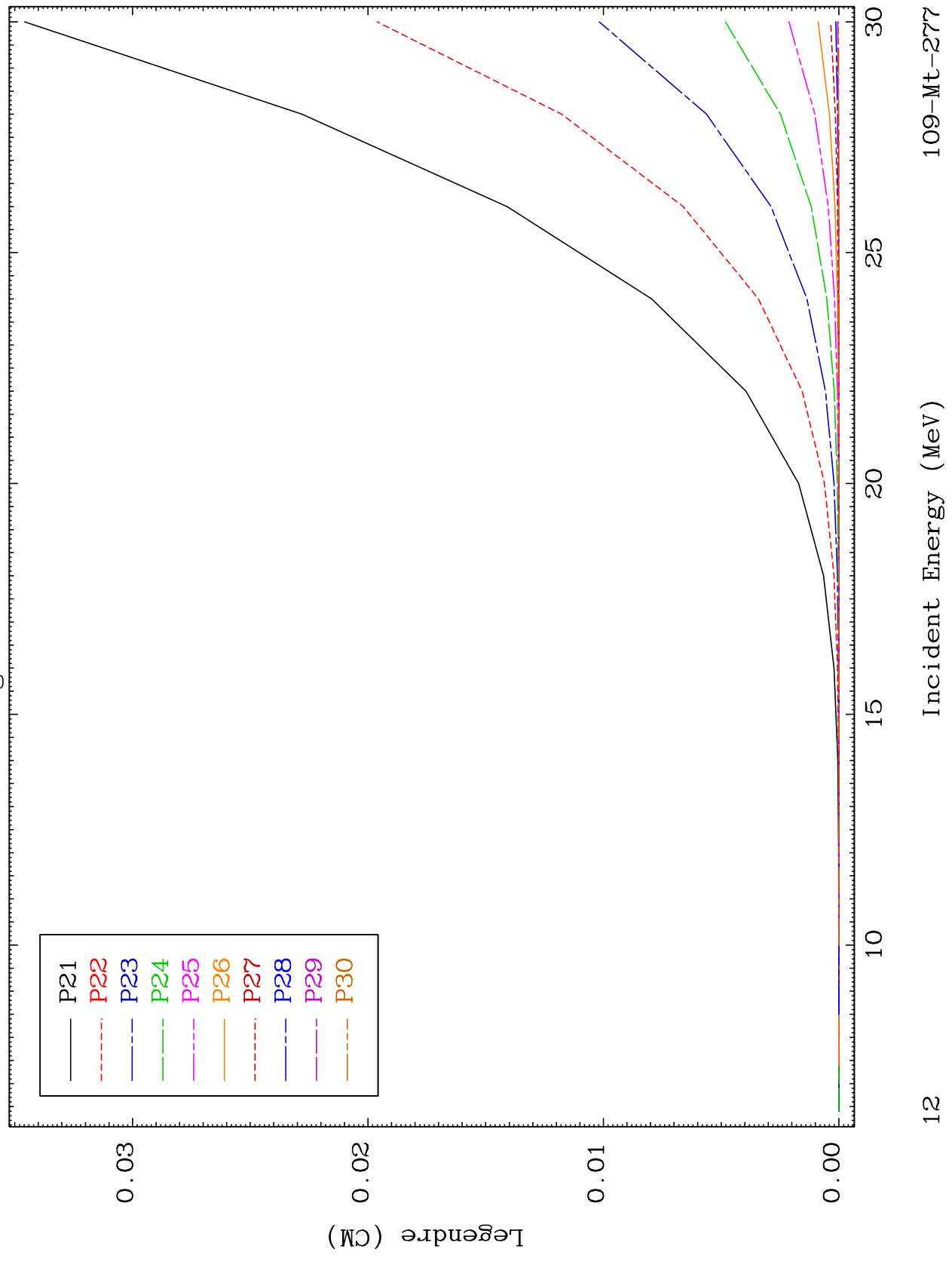




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Elastic Legendre Coefficients

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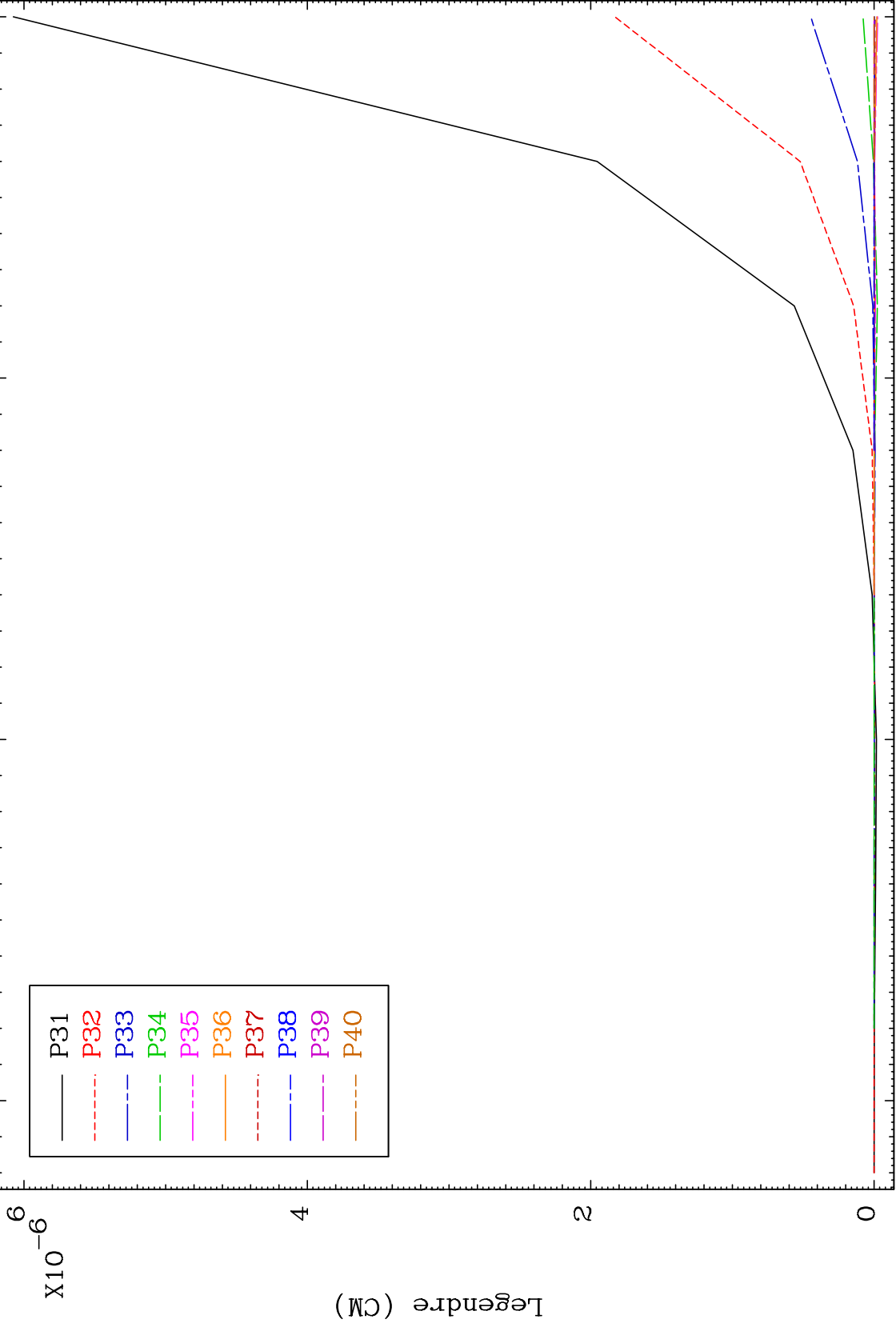
12

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Elastic
Legendre Coefficients

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13

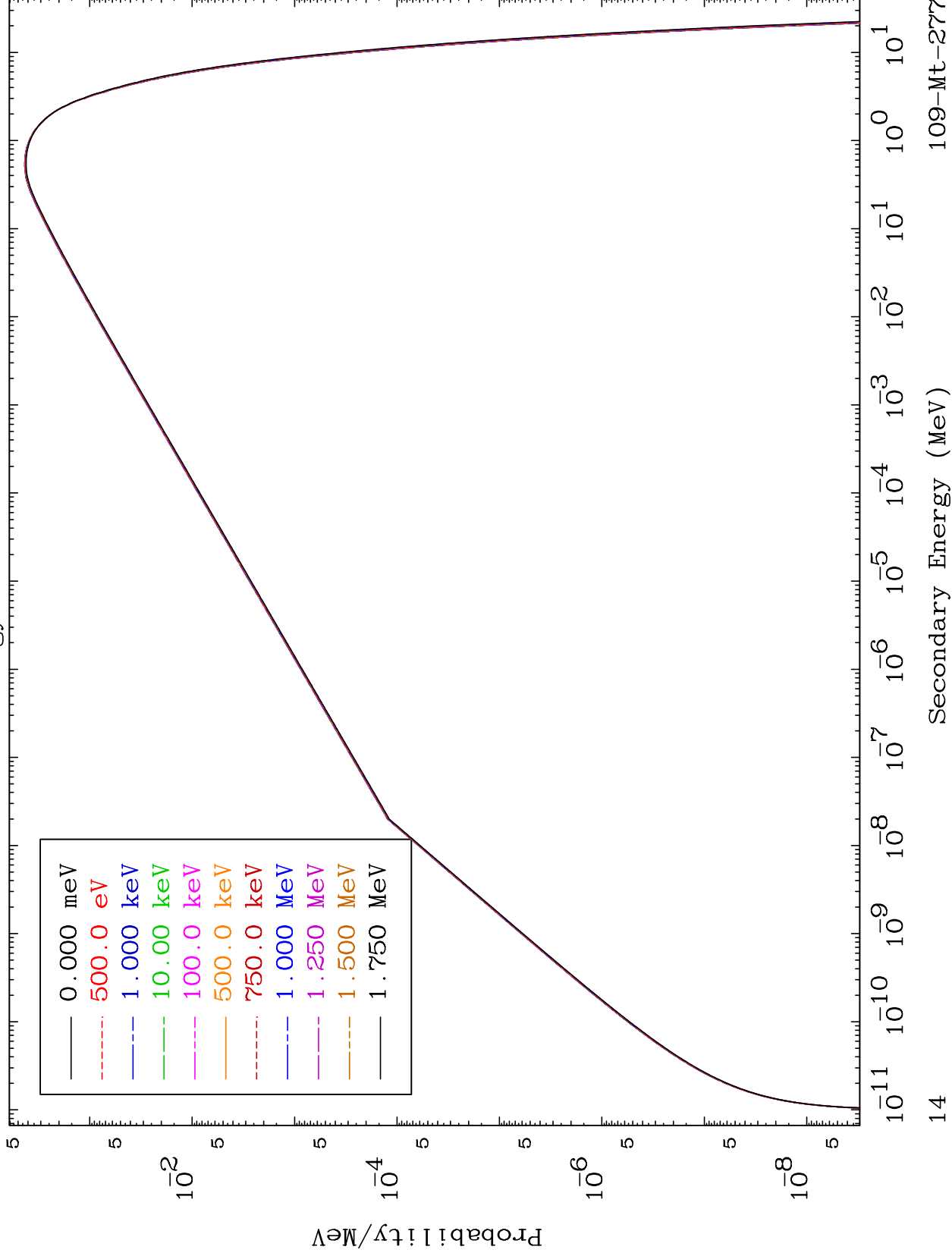
Incident Energy (MeV)

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Fission Energy Distributions

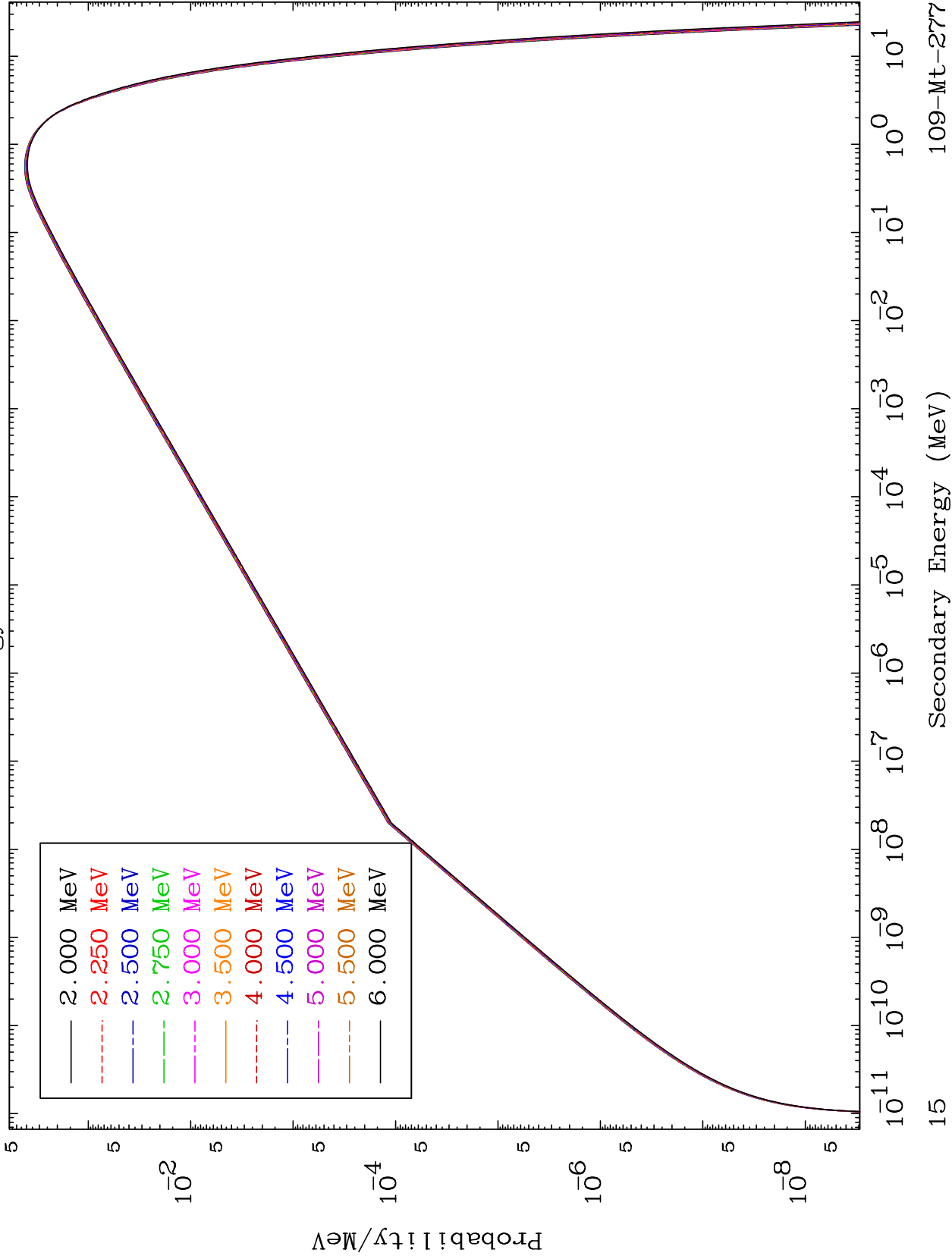
109-Mt-277



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Fission Energy Distributions

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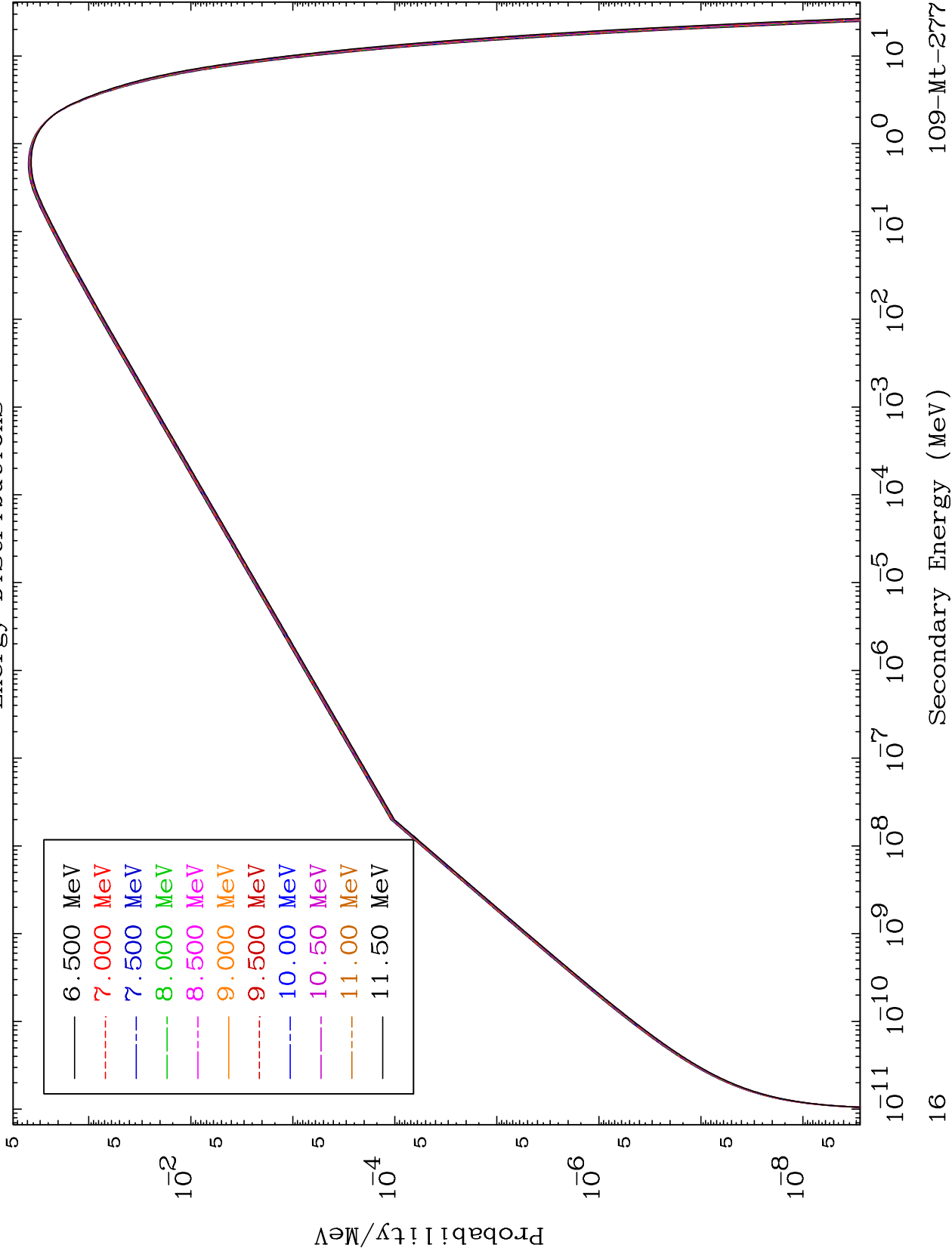
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Secondary Energy (MeV)

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Fission Energy Distributions

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16

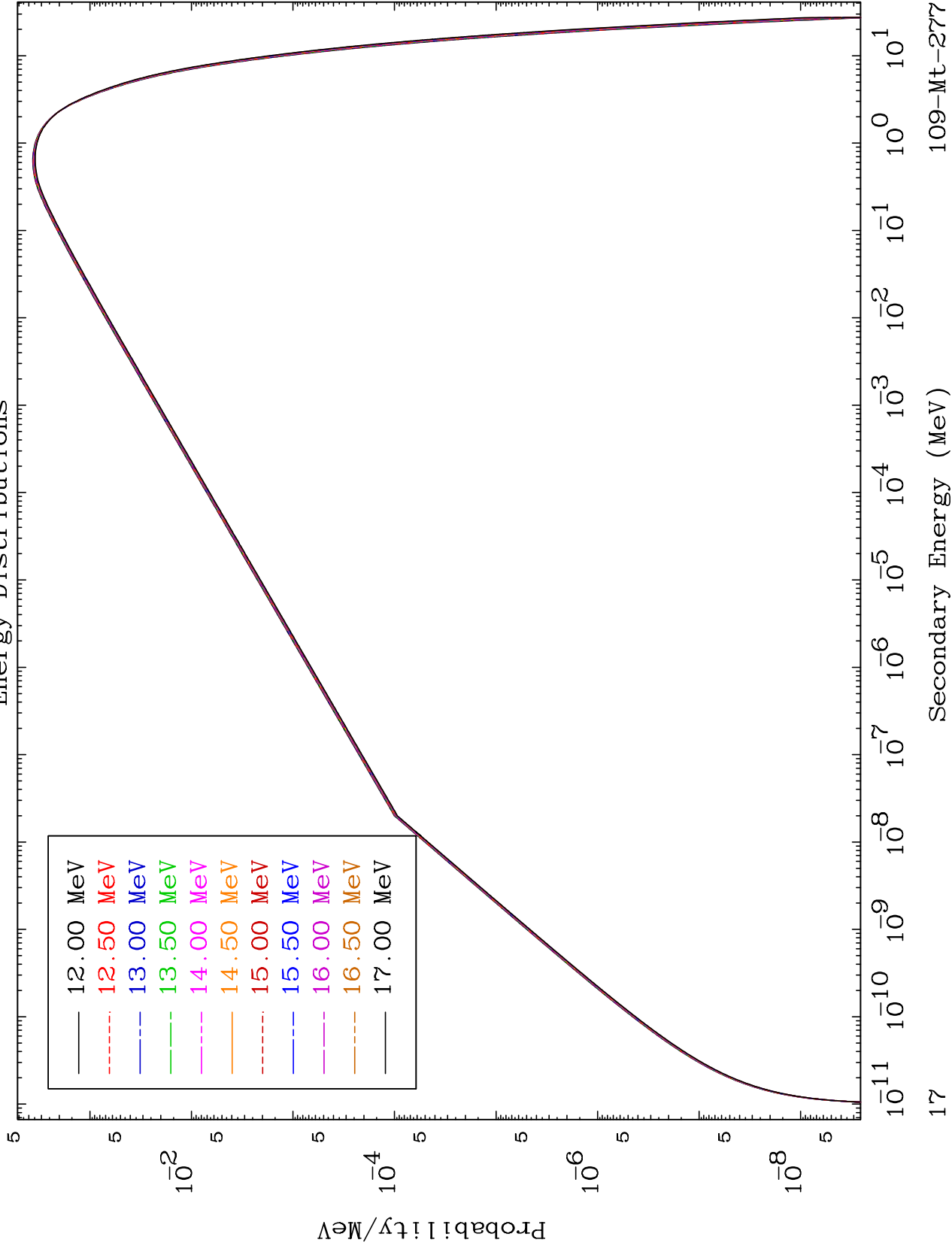
Secondary Energy (MeV)

109-Mt-277

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Fission Energy Distributions

109-Mt-277



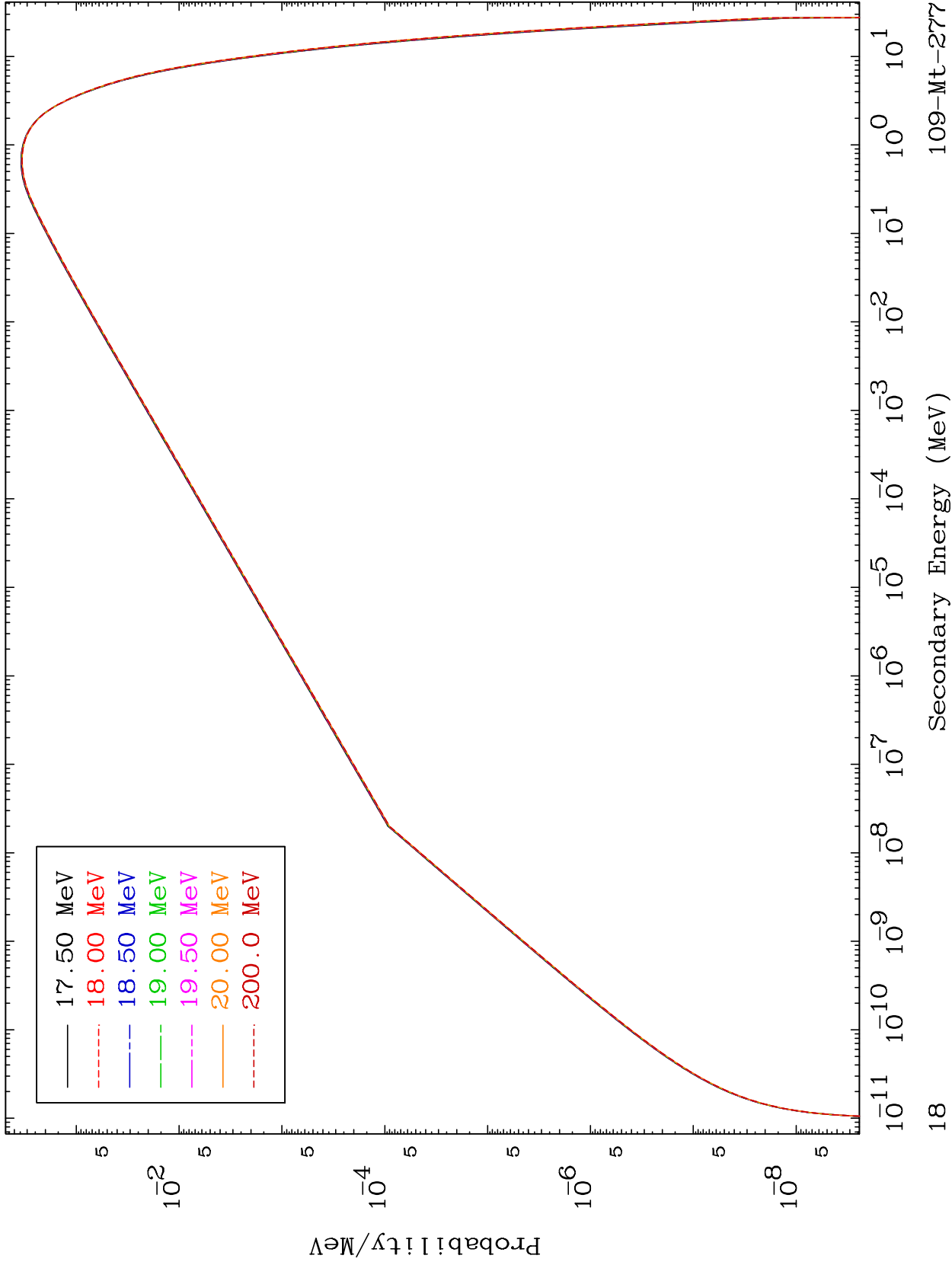
17

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Fission Energy Distributions

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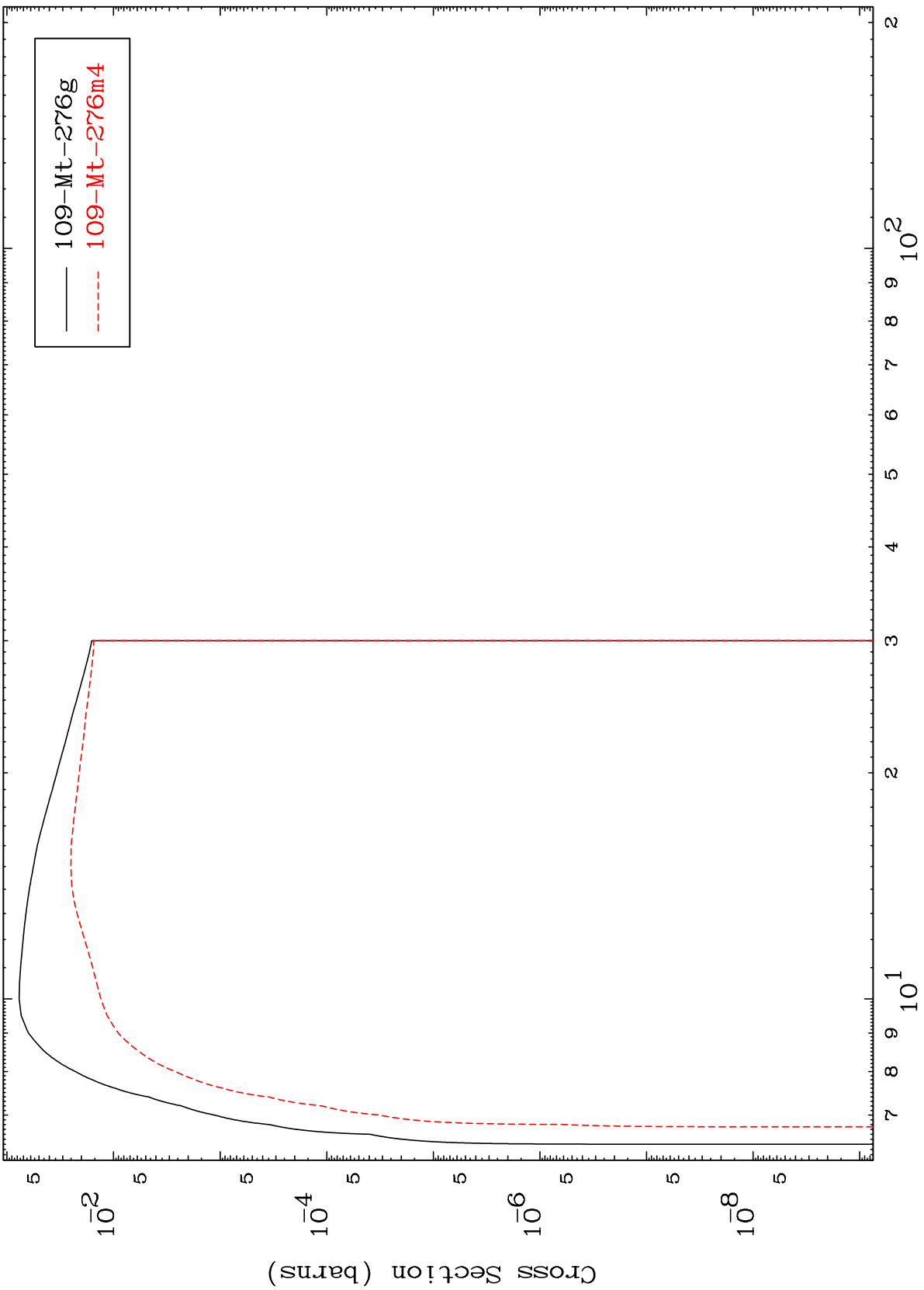


18

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(n,2n)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

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