

Program EVALPLOT
(Version 2021-1)

by

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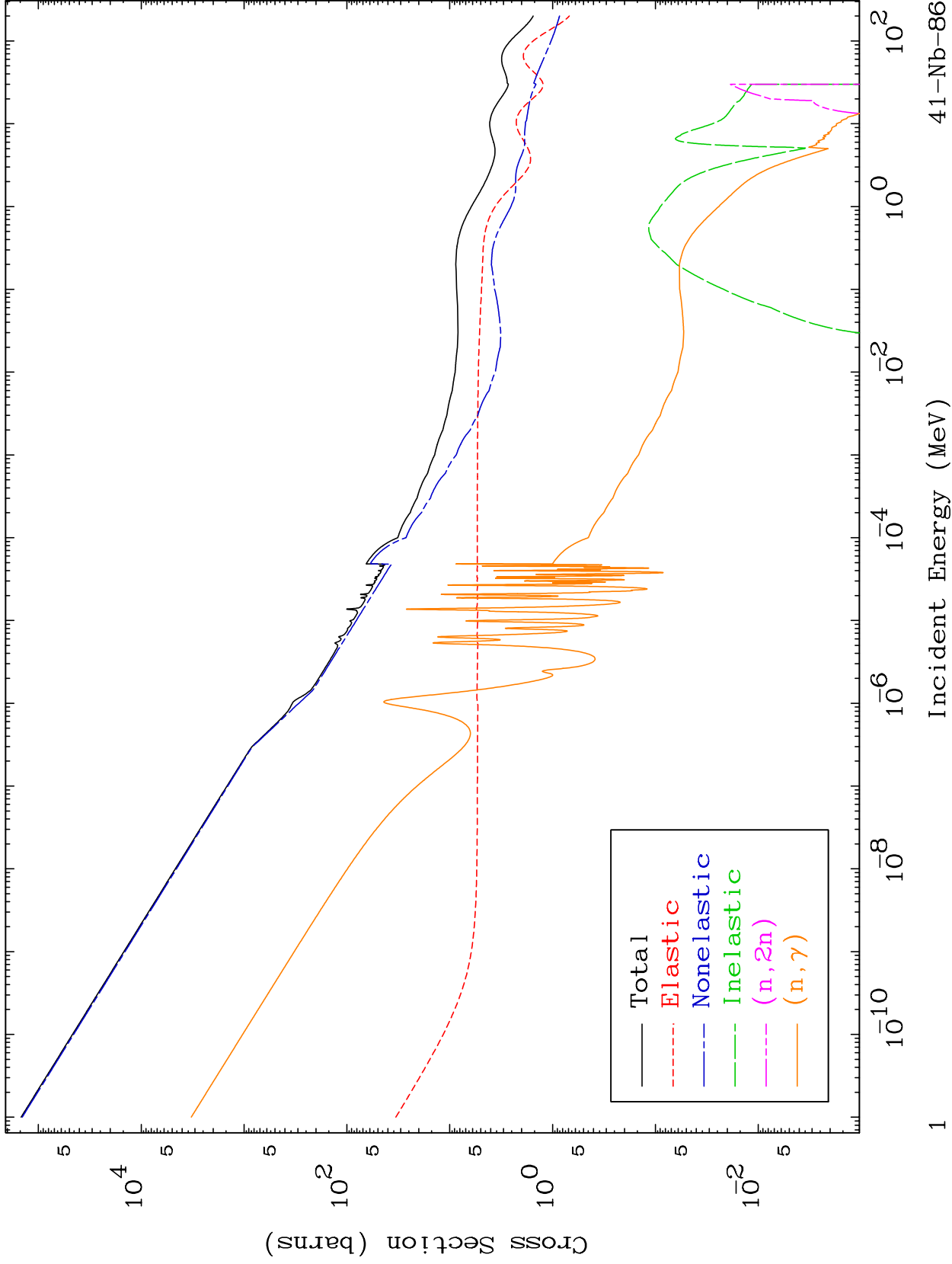
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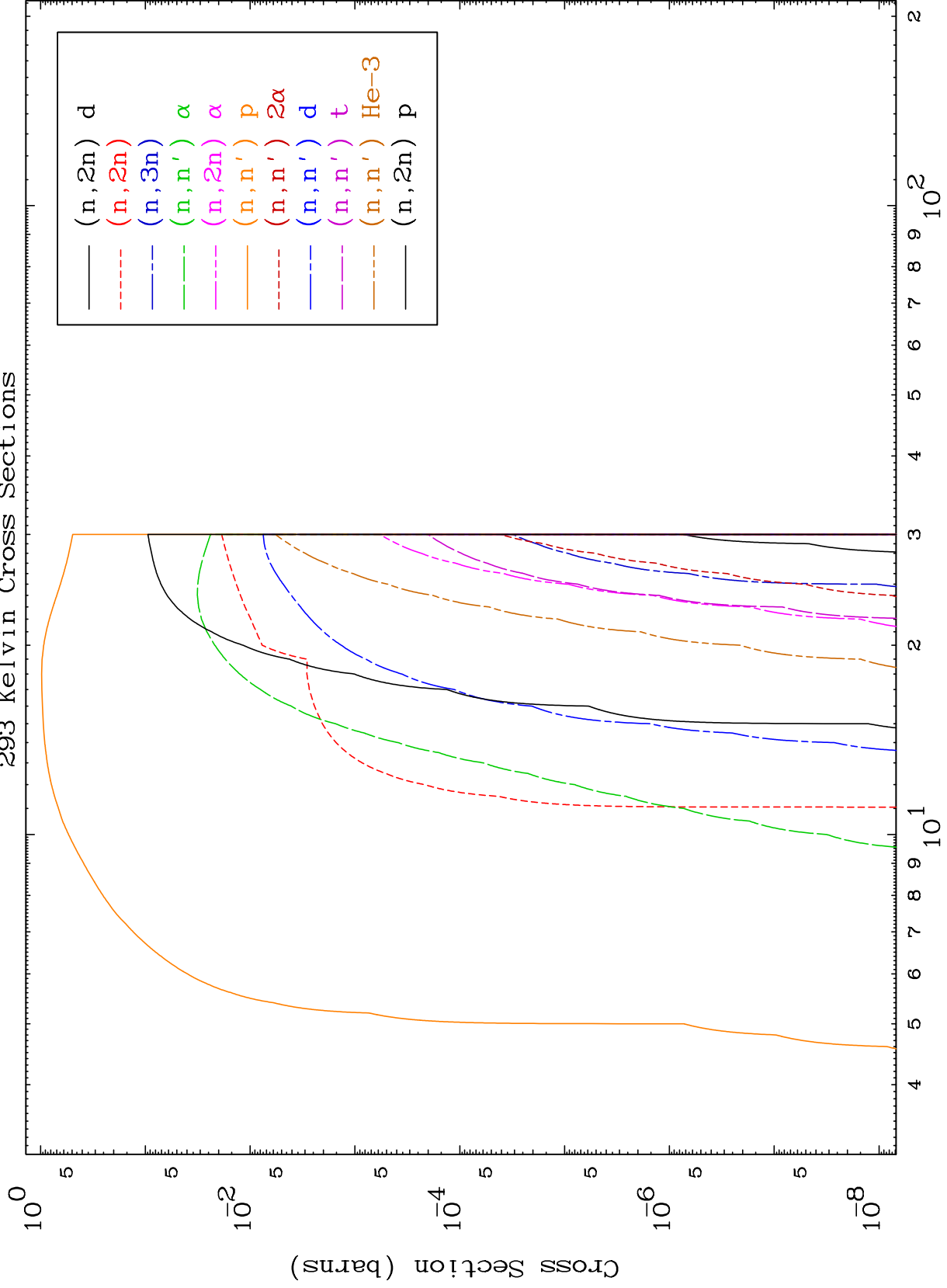
Press Mouse Button to Start

MAT 4104

Neutron Major
293 Kelvin Cross Sections

41-Nb-86

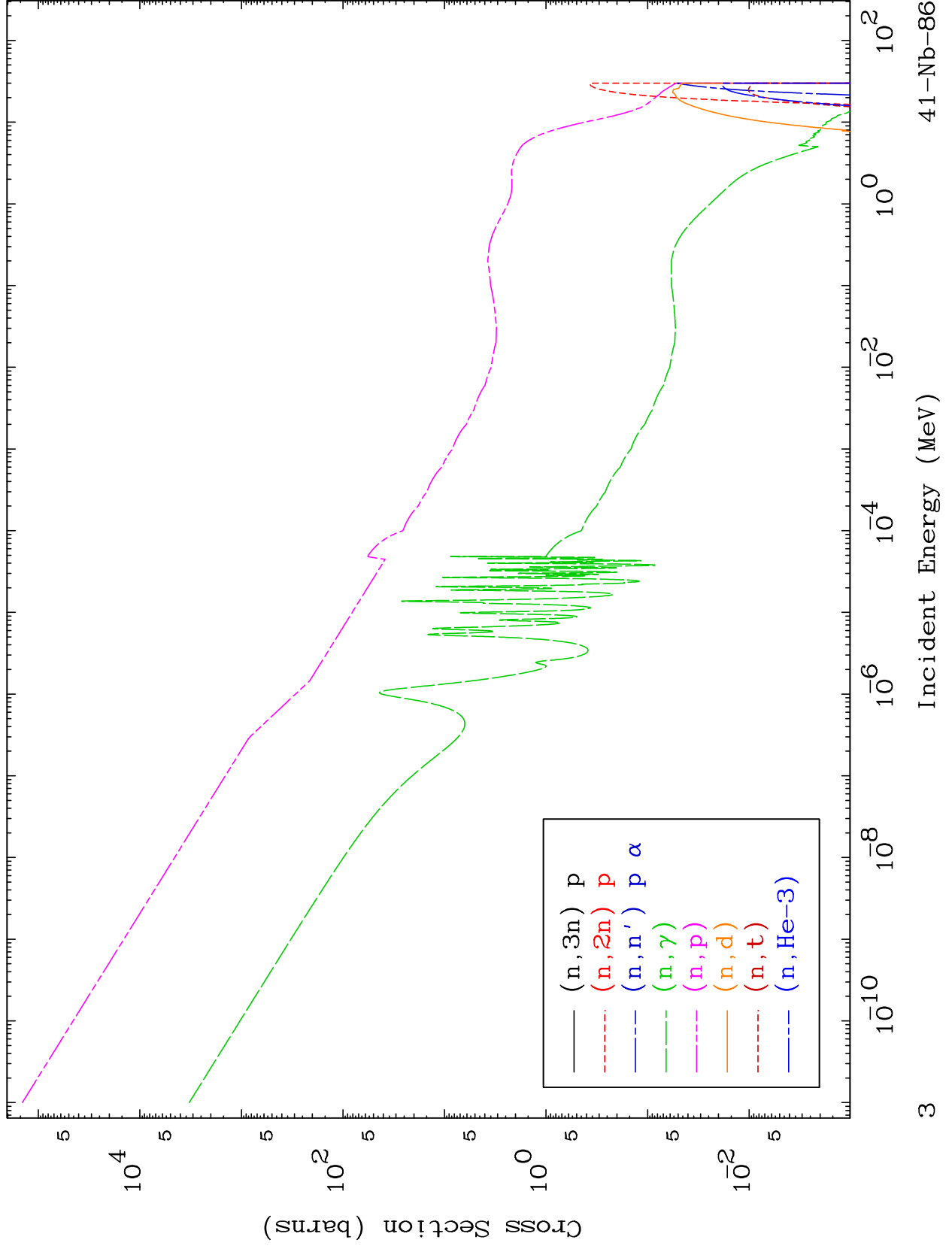


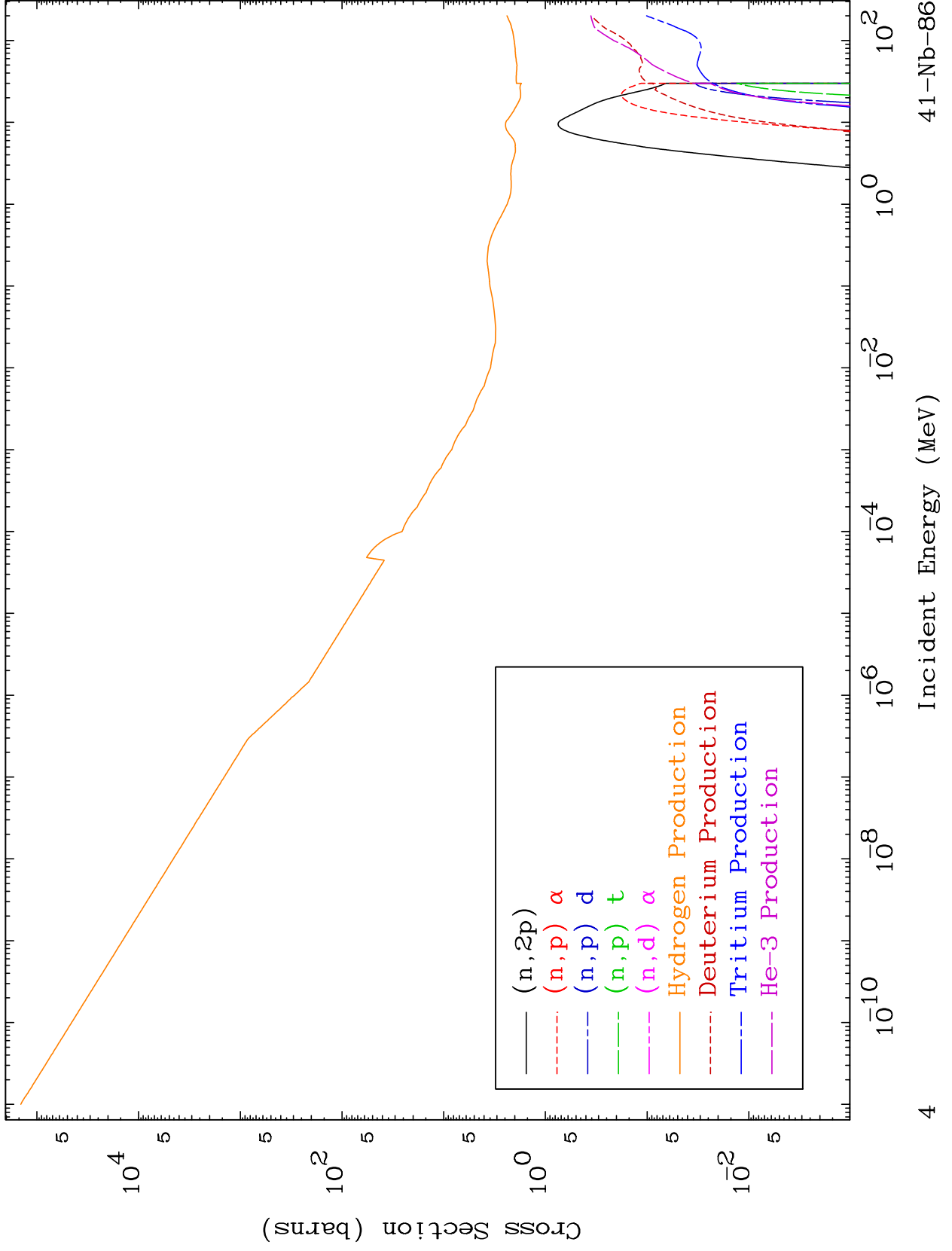


MAT 4104

Neutron Absorption
293 Kelvin Cross Sections

41-Nb-86

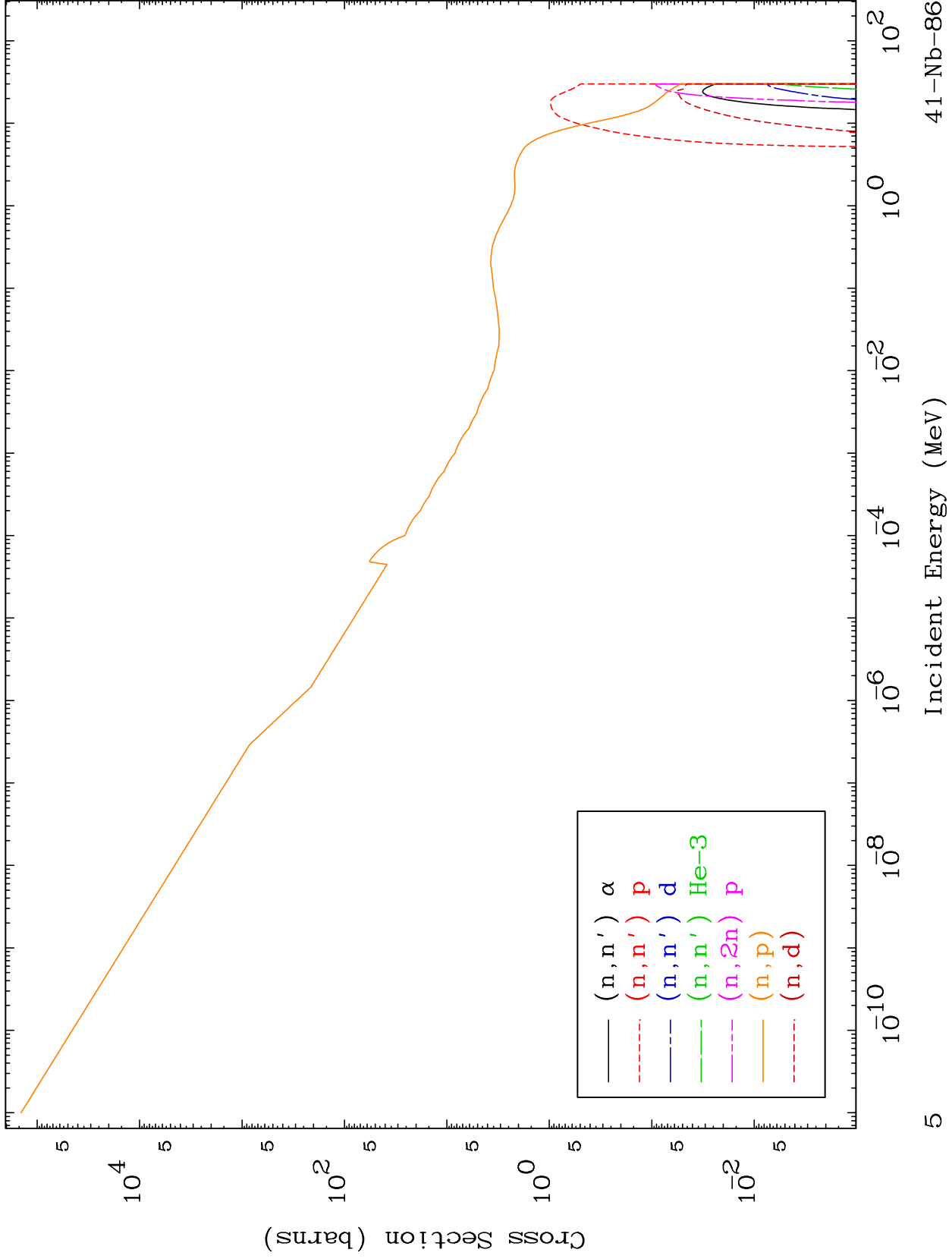




MAT 4104

Charged Particle
293 Kelvin Cross Sections

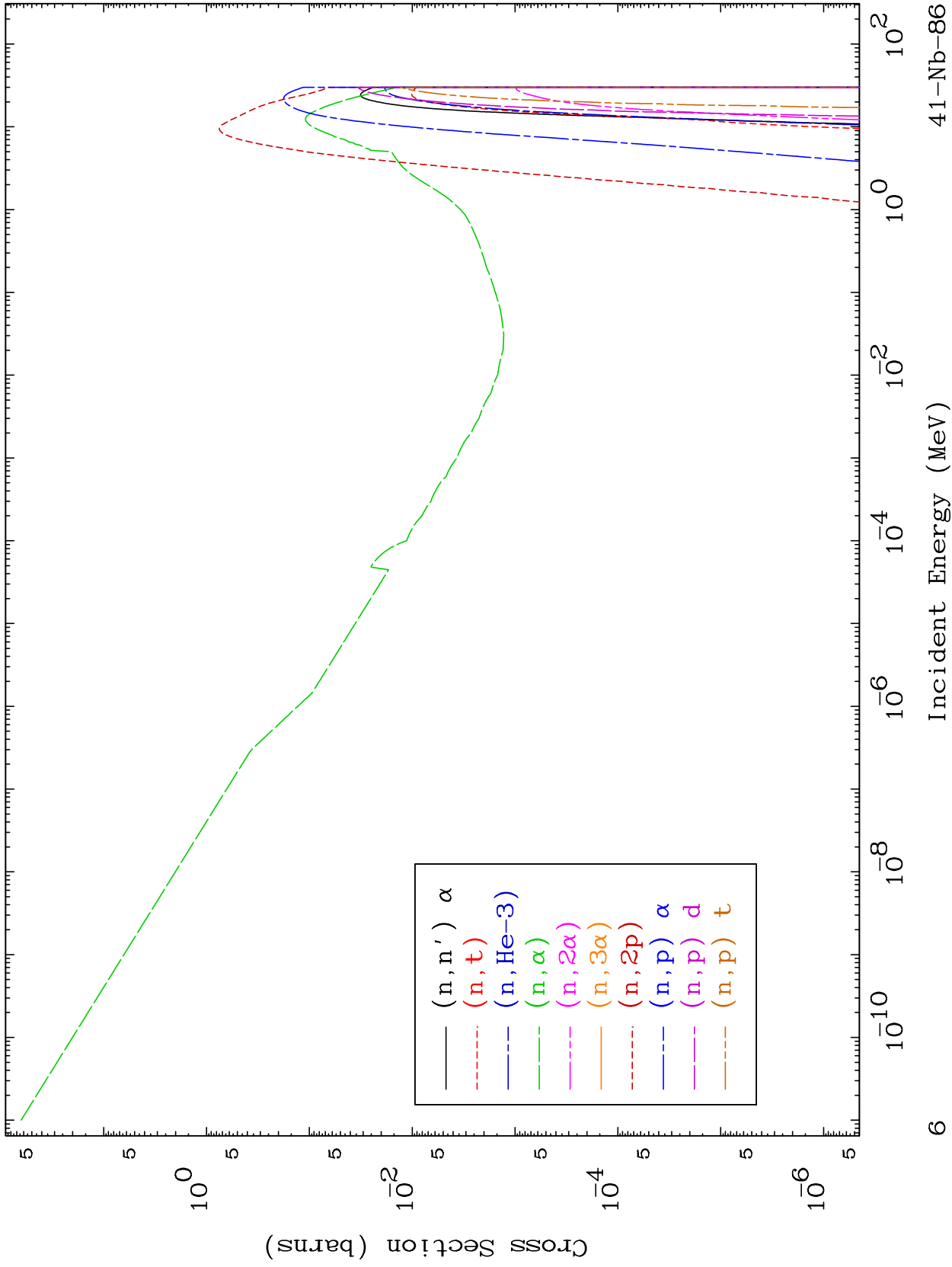
41-Nb-86



MAT 4104

Charged Particle
293 Kelvin Cross Sections

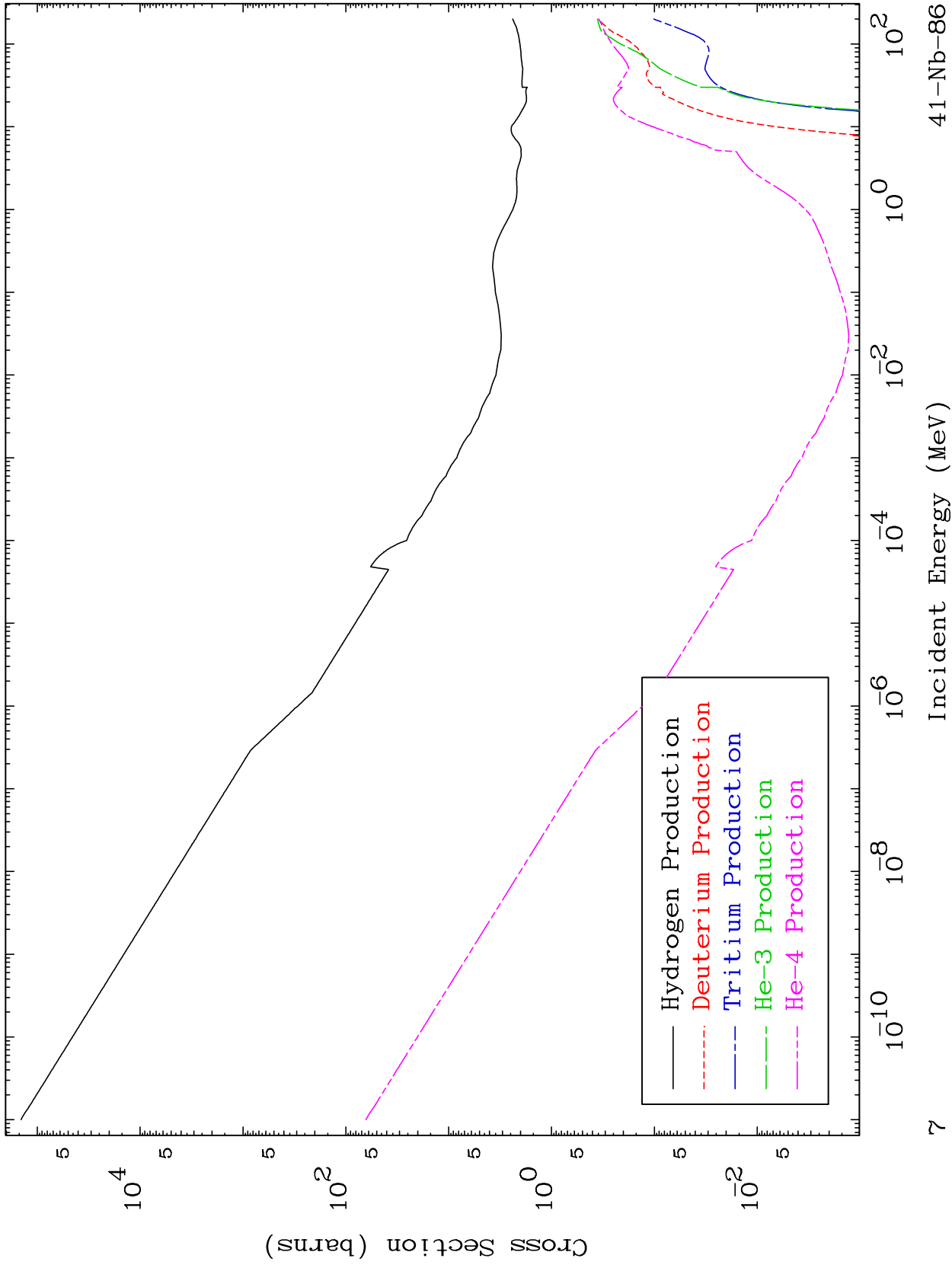
41-Nb-86



MAT 4104

Particle Production
293 Kelvin Cross Sections

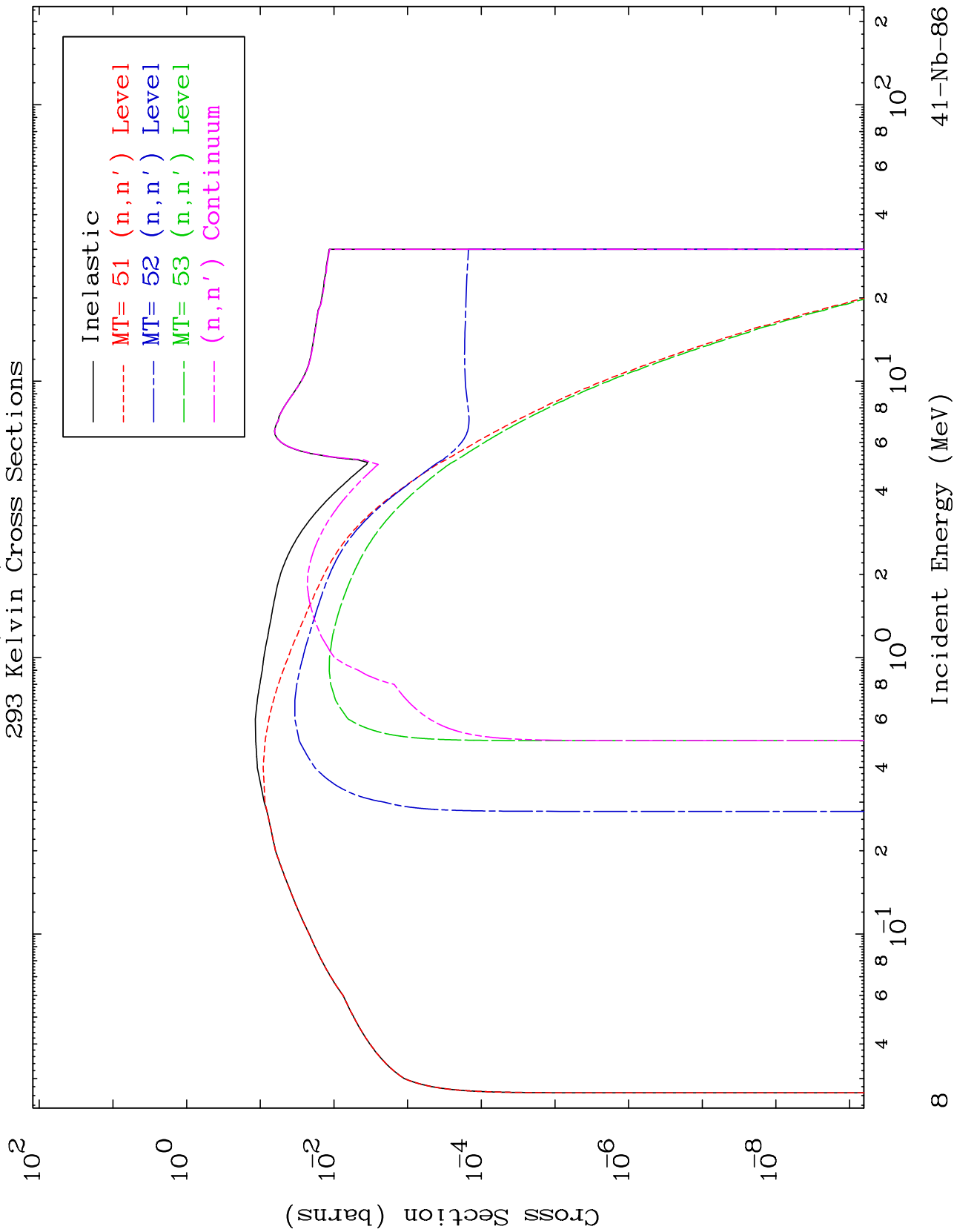
41-Nb-86



MAT 4104

(n,n') Levels
293 Kelvin Cross Sections

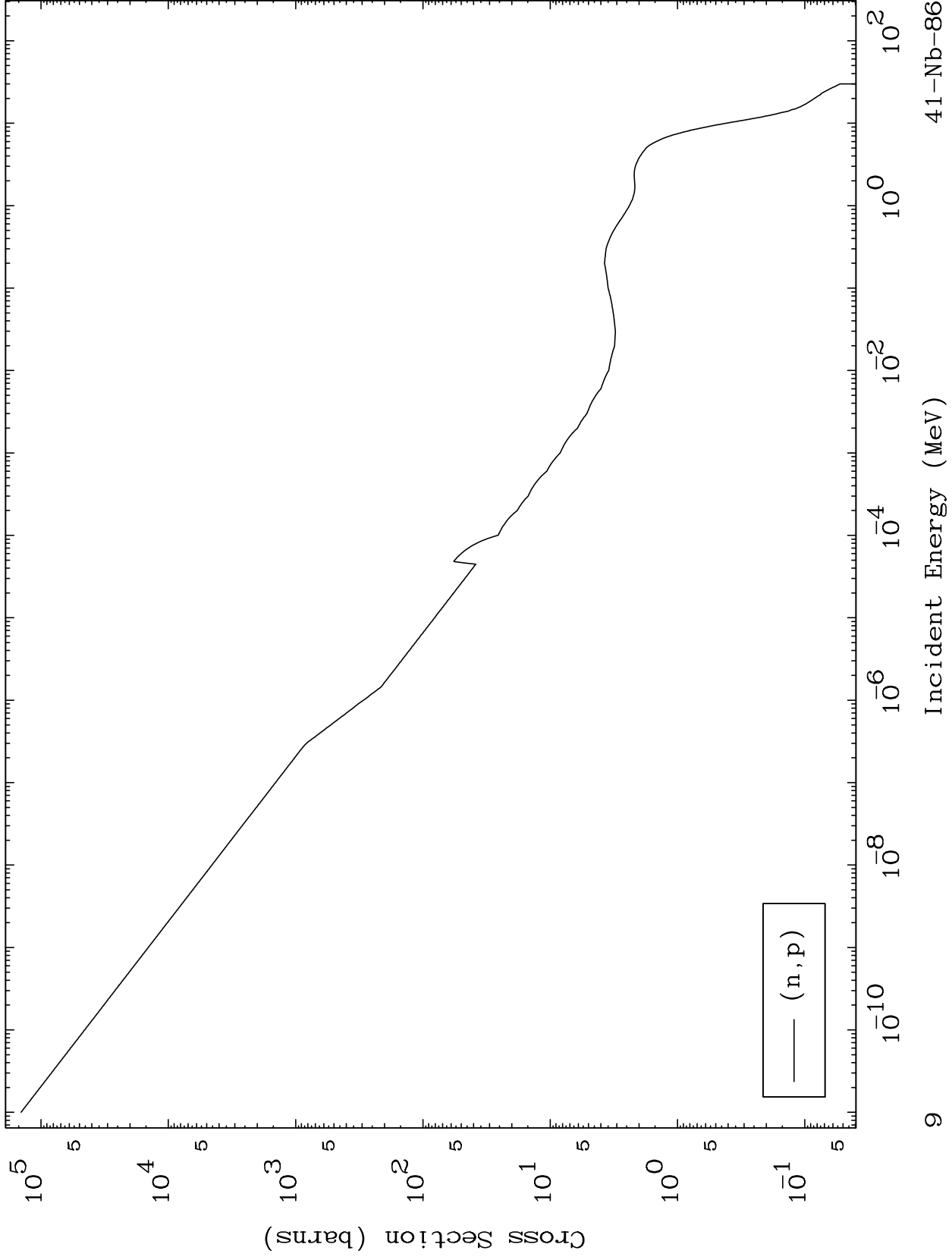
41-Nb-86



MAT 4104

(n,p) Levels
293 Kelvin Cross Sections

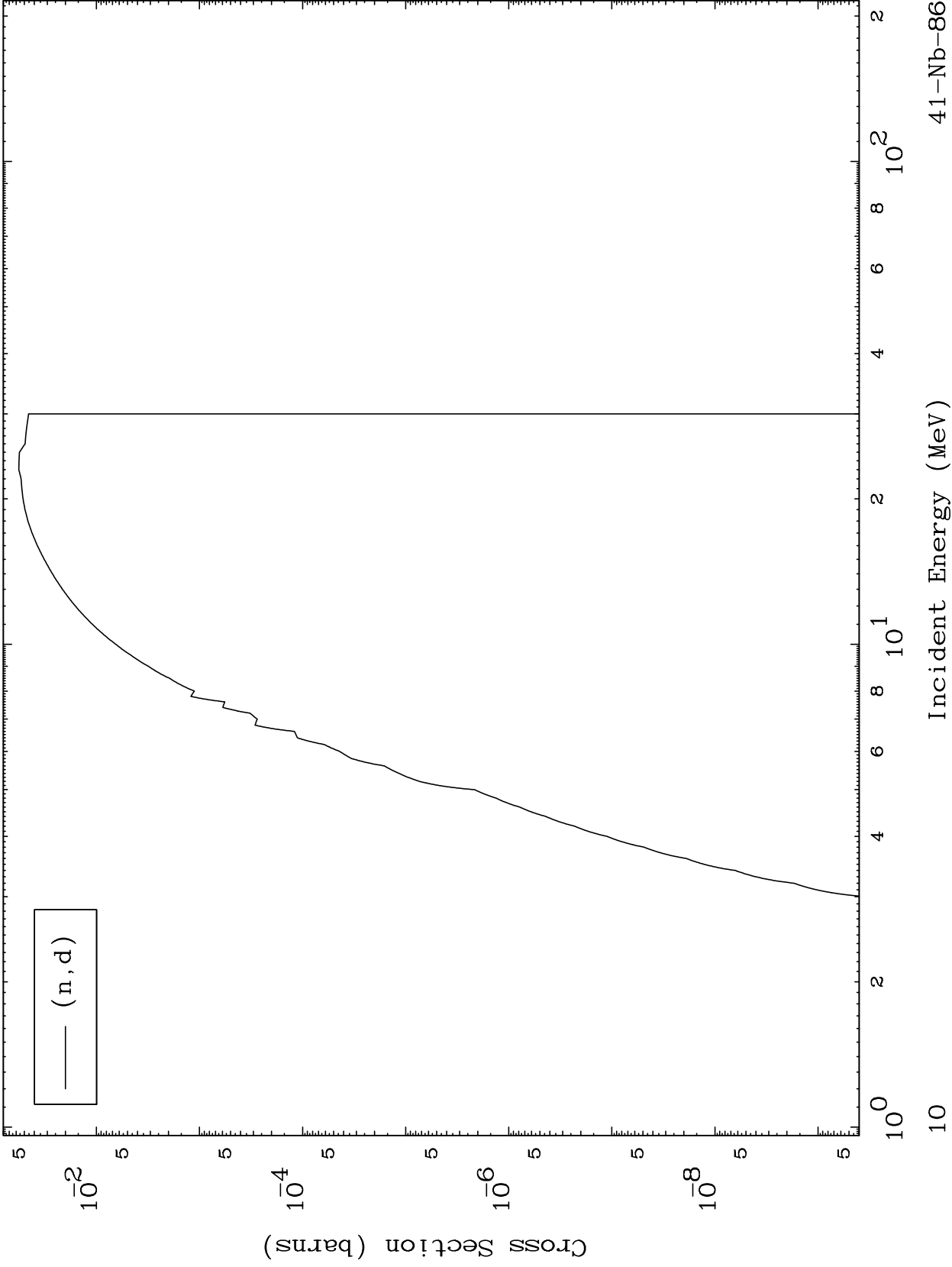
41-Nb-86



MAT 4104

(n,d) Levels
293 Kelvin Cross Sections

41-Nb-86

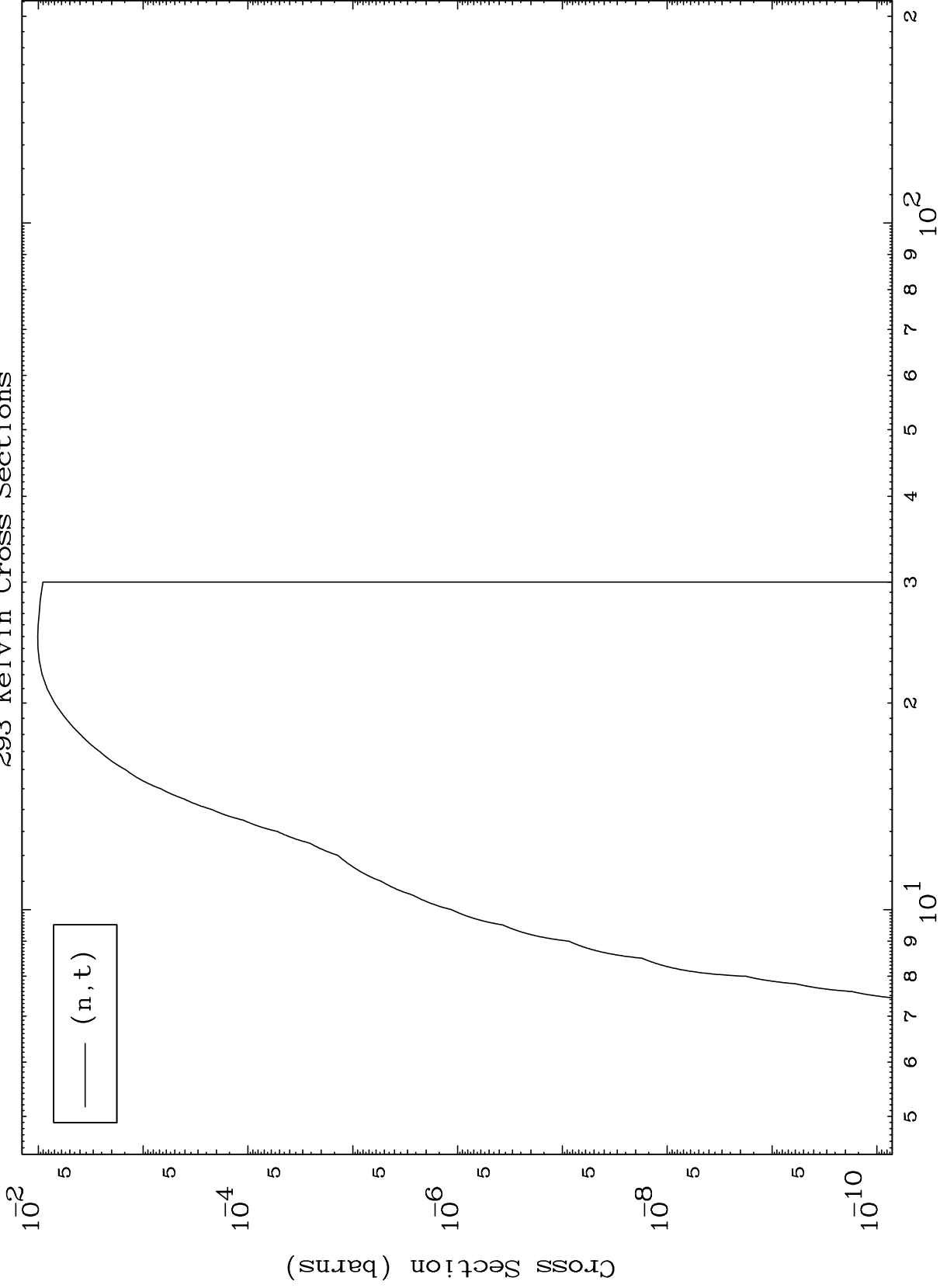


41-Nb-86

MAT 4104

(n,t) Levels
293 Kelvin Cross Sections

41-Nb-86



11

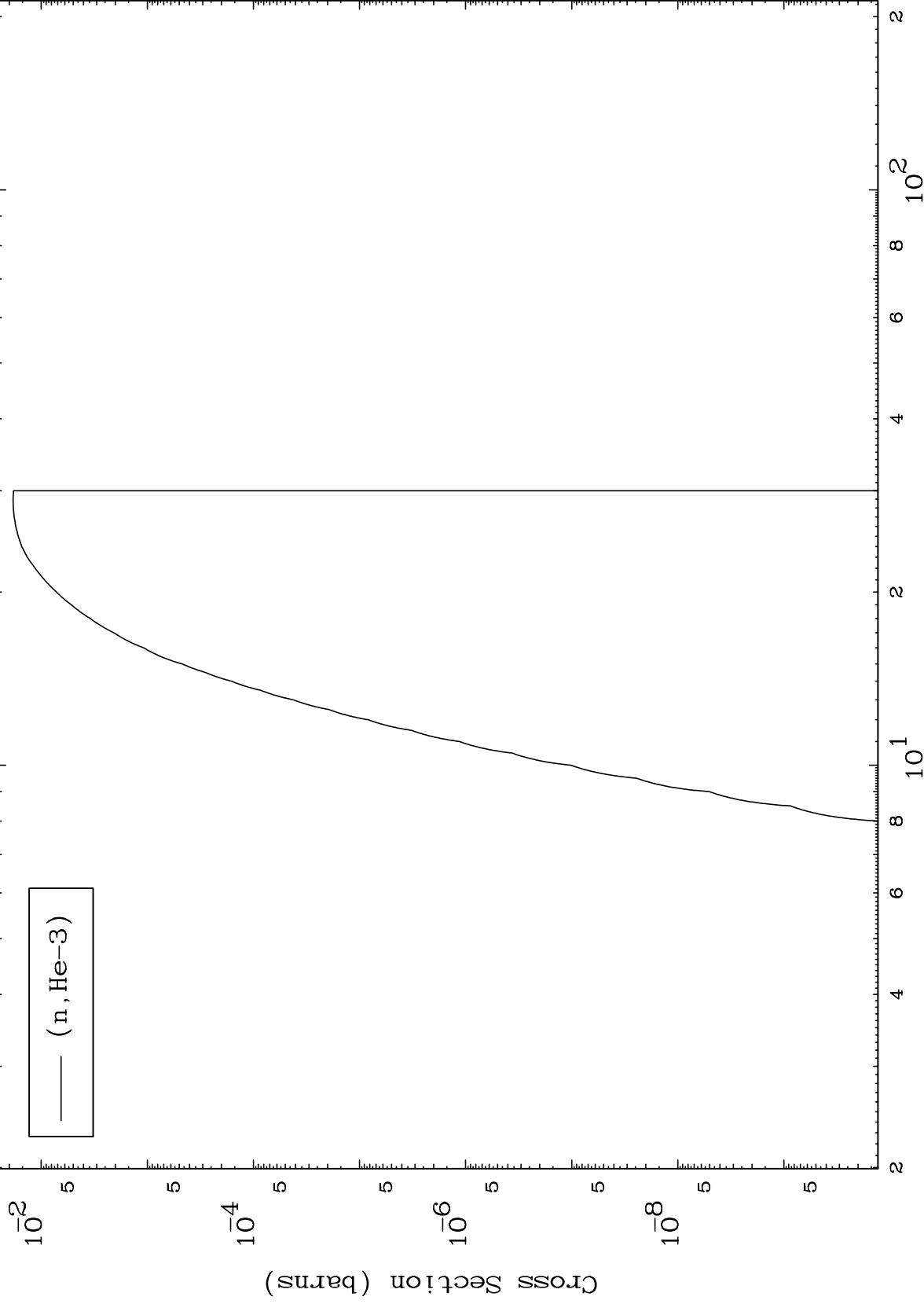
Incident Energy (MeV)

41-Nb-86

MAT 4104

(n,He3) Levels
293 Kelvin Cross Sections

41-Nb-86



12

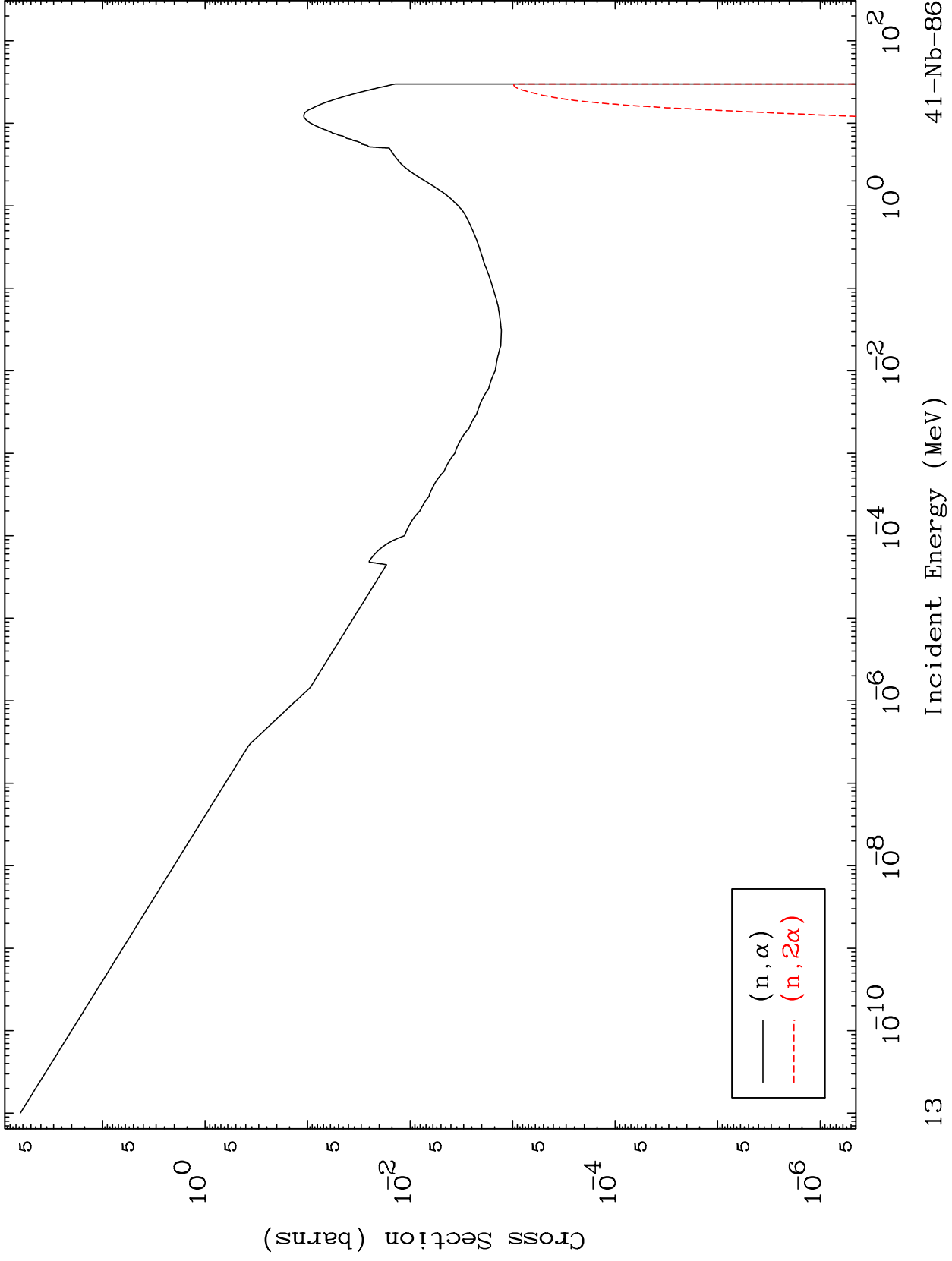
Incident Energy (MeV)

41-Nb-86

MAT 4104

(n, α) Levels
293 Kelvin Cross Sections

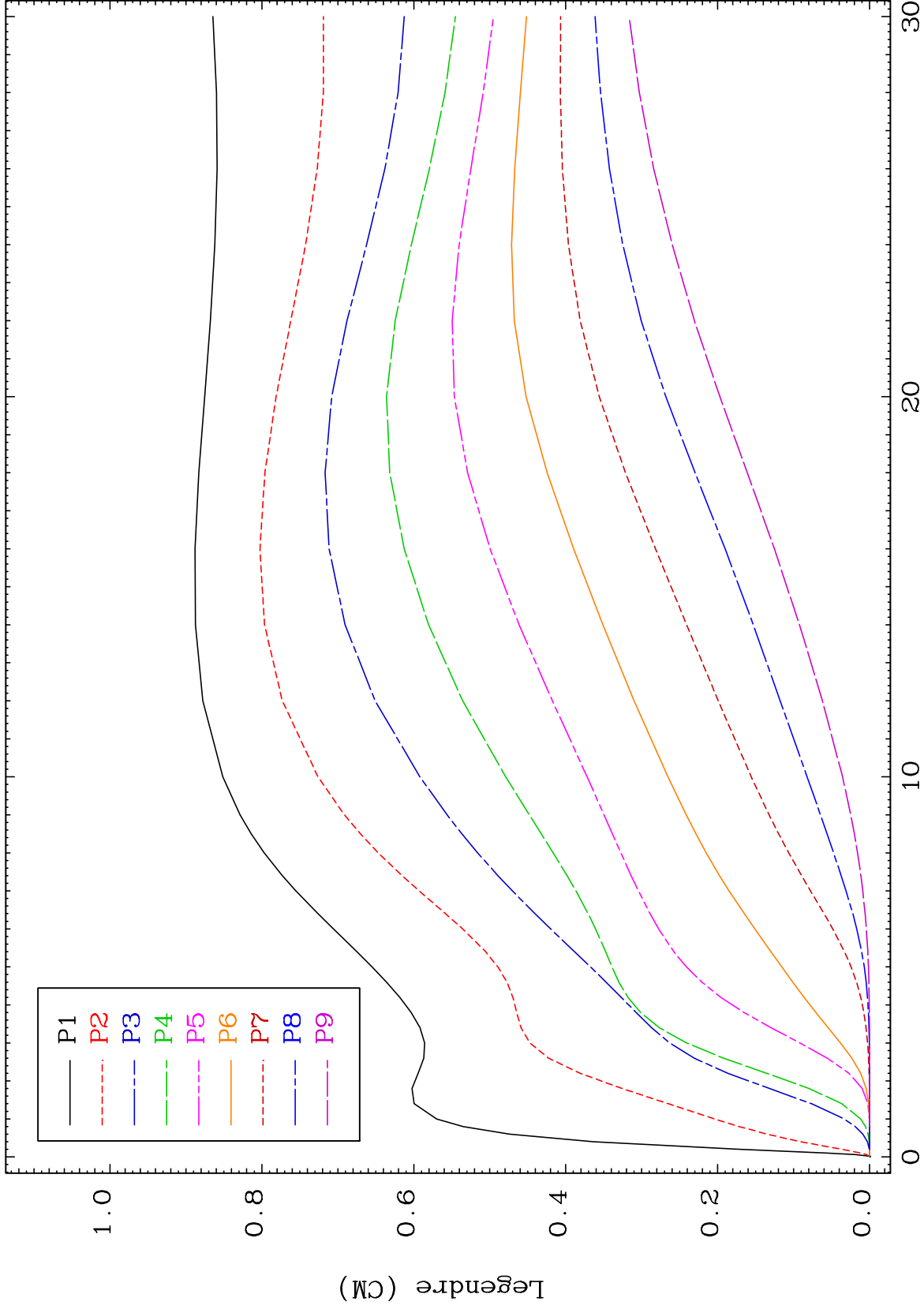
41-Nb-86



MAT 4104

Elastic Legendre Coefficients

41-Nb-86



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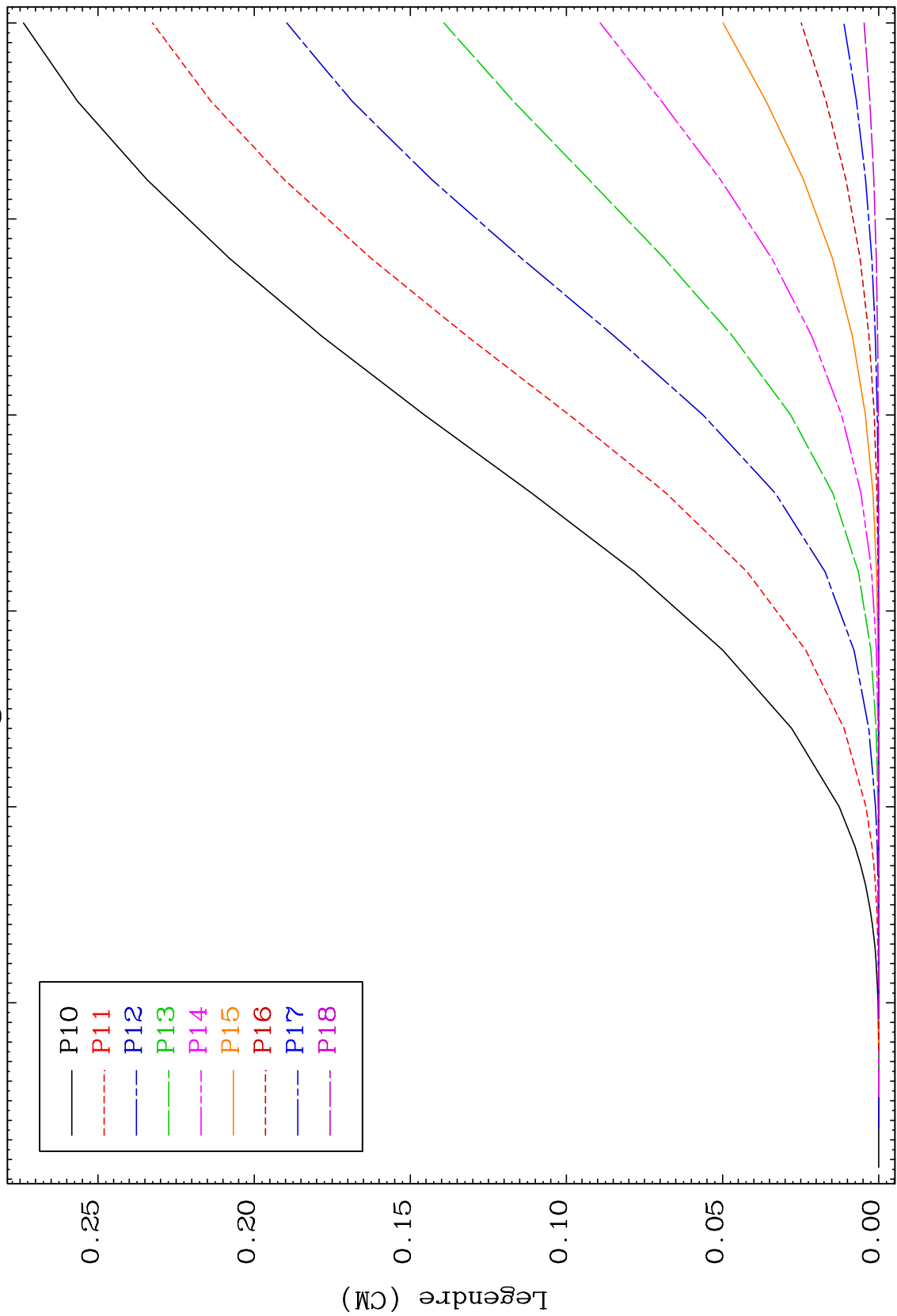
Incident Energy (MeV)

41-Nb-86

MAT 4104

Elastic Legendre Coefficients

41-Nb-86



15

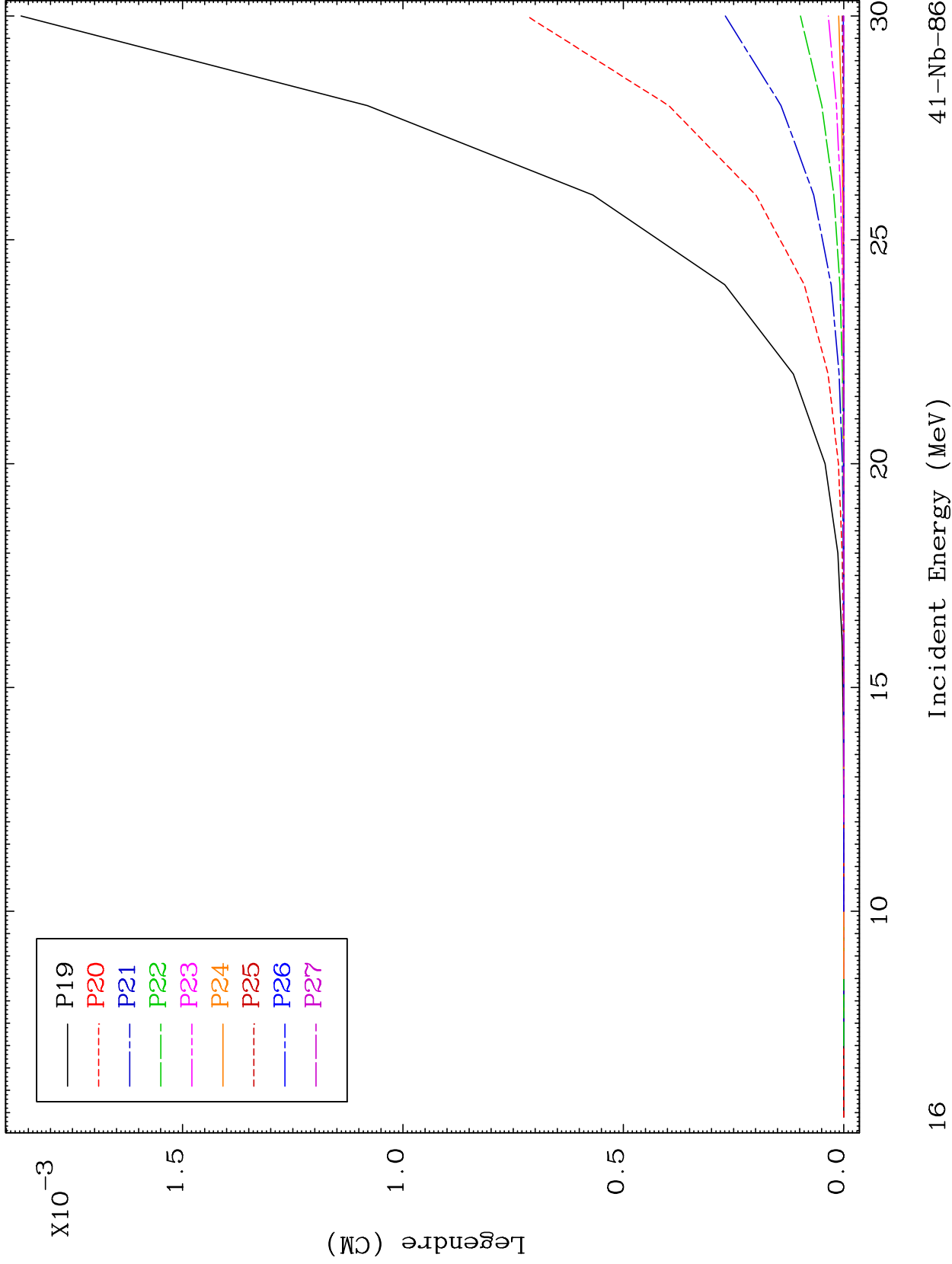
Incident Energy (MeV)

41-Nb-86

MAT 4104

Elastic Legendre Coefficients

41-Nb-86



16

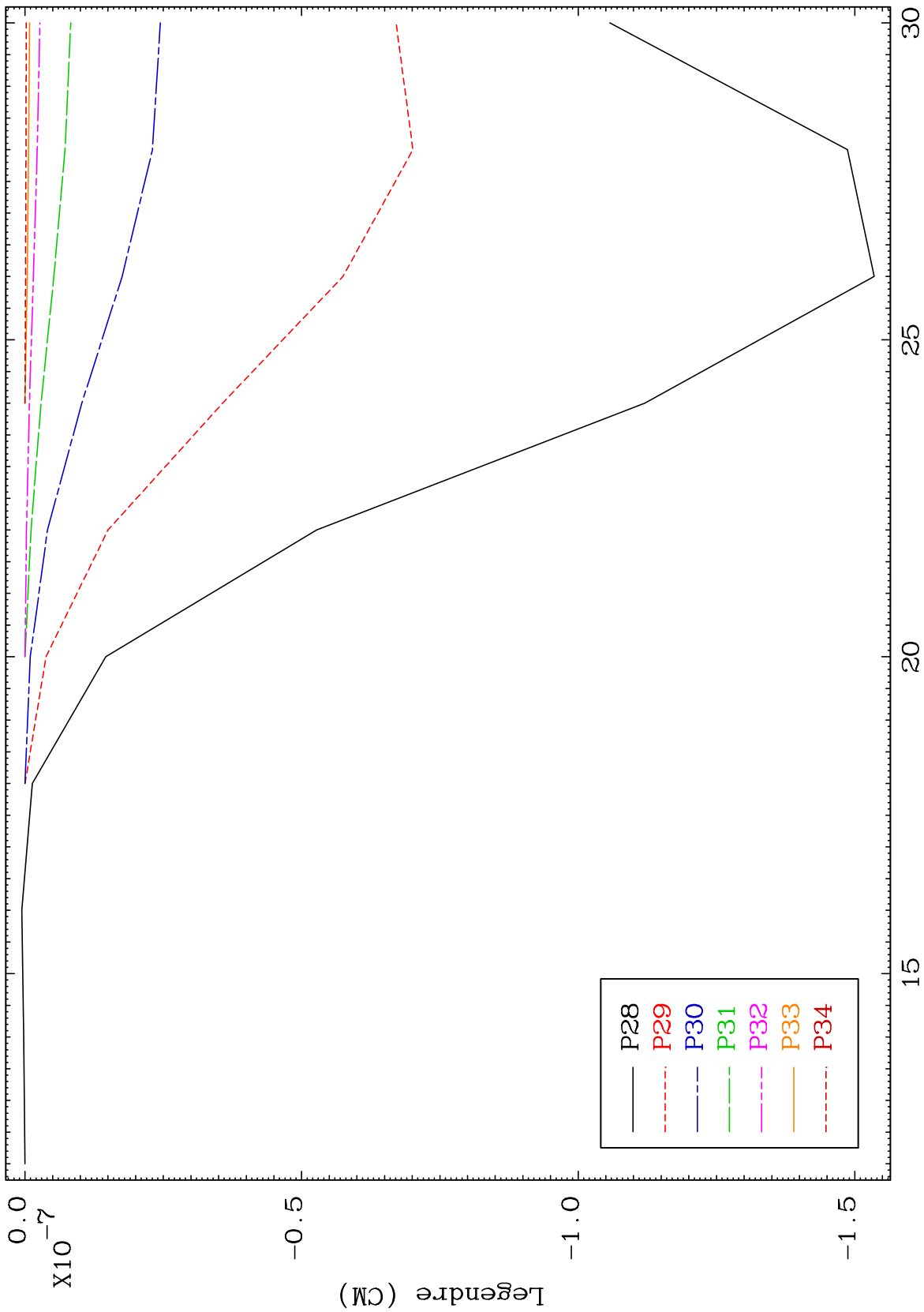
Incident Energy (MeV)

41-Nb-86

MAT 4104

41-Nb-86

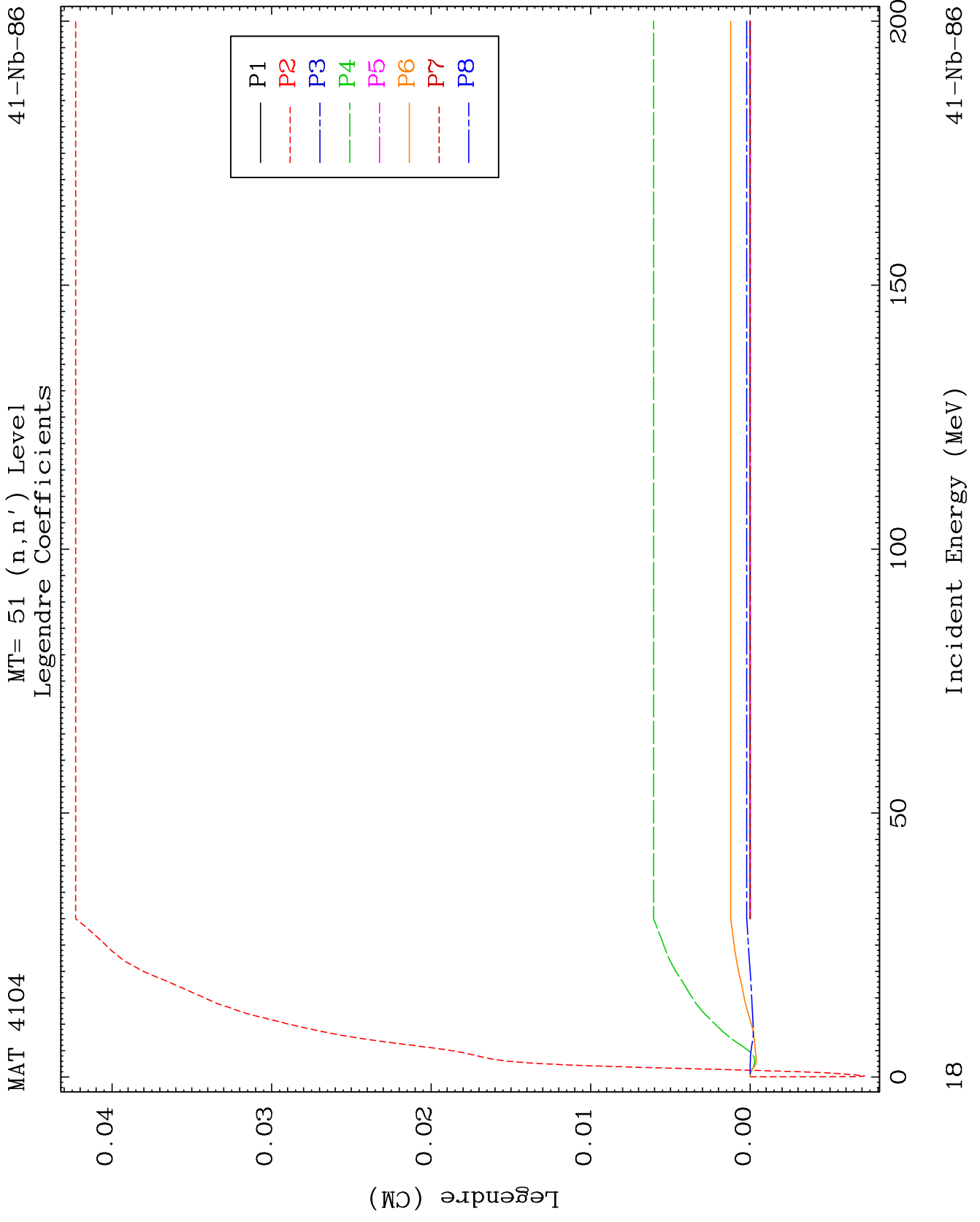
Elastic Legendre Coefficients



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Incident Energy (MeV)

41-Nb-86

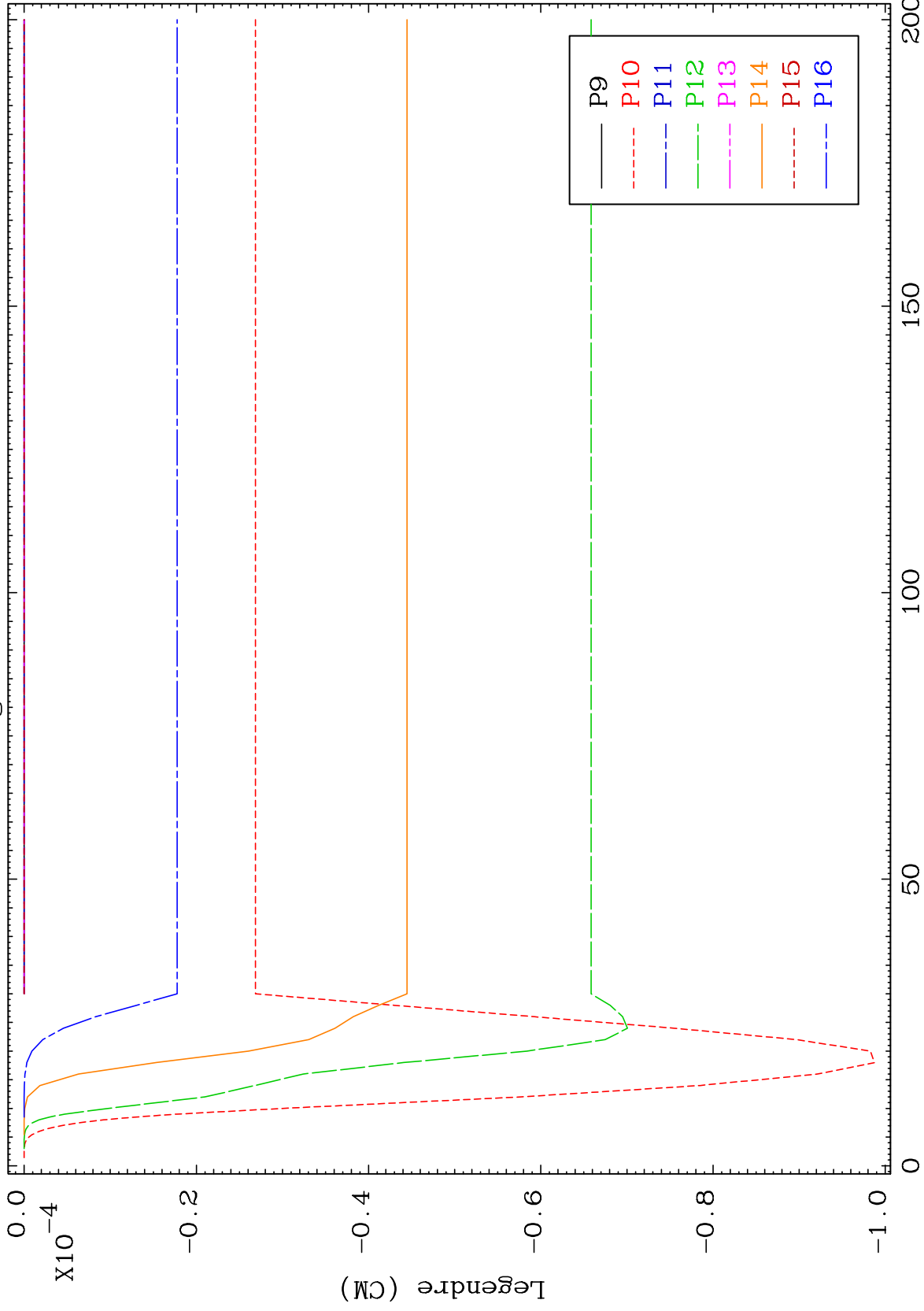


MAT 4104

MT= 51 (n,n') Level

41-Nb-86

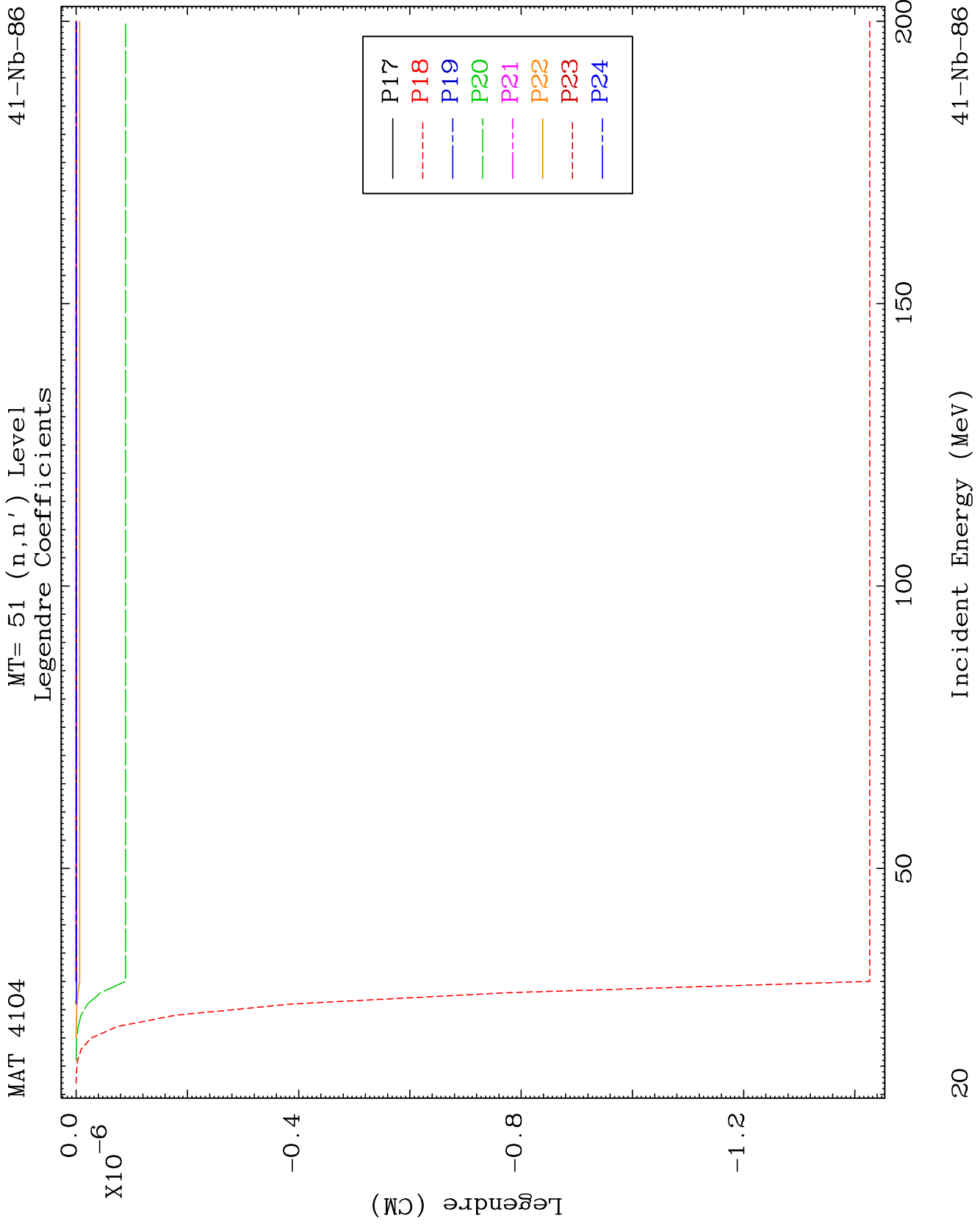
Legendre Coefficients

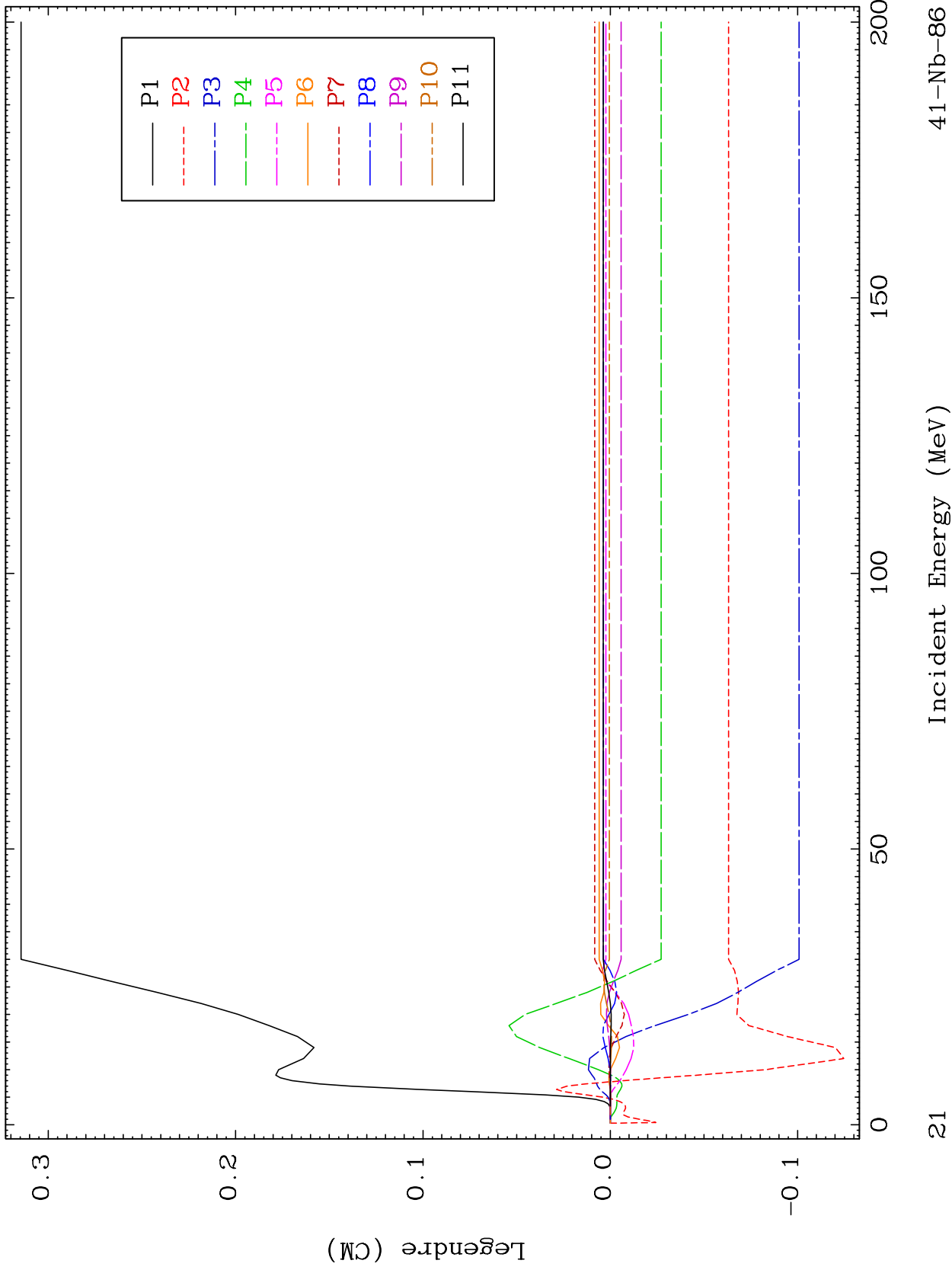


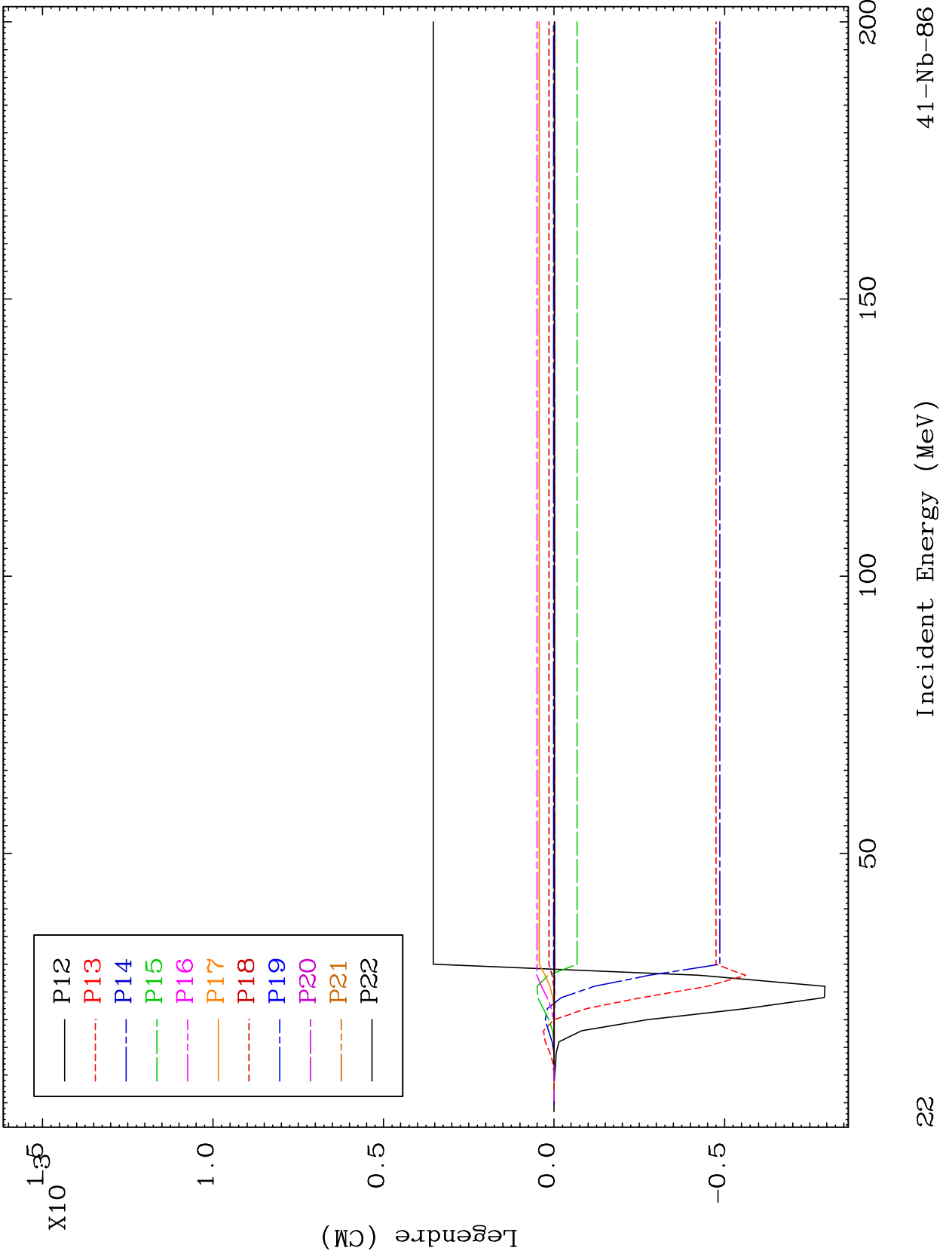
19

Incident Energy (MeV)

41-Nb-86



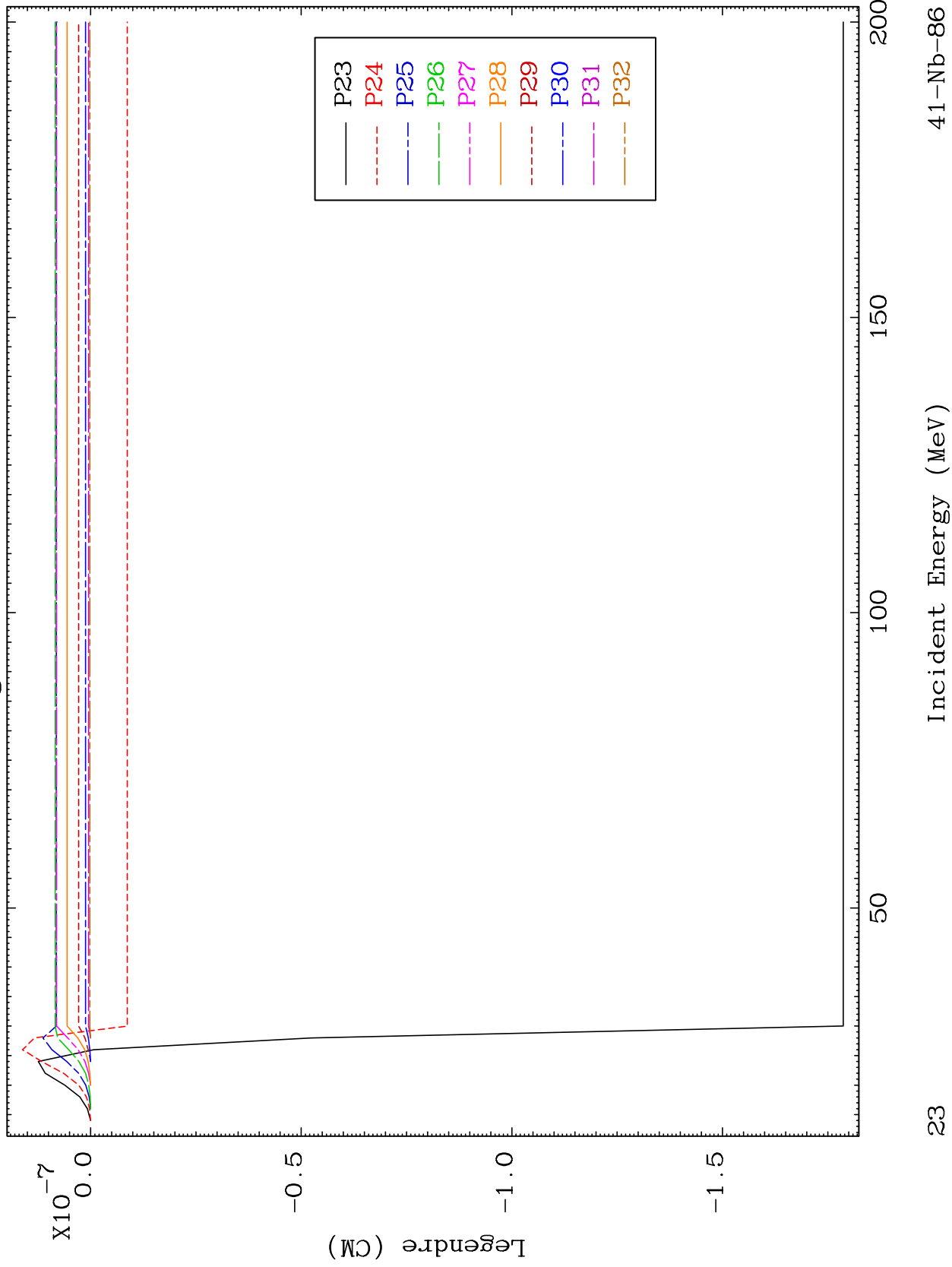




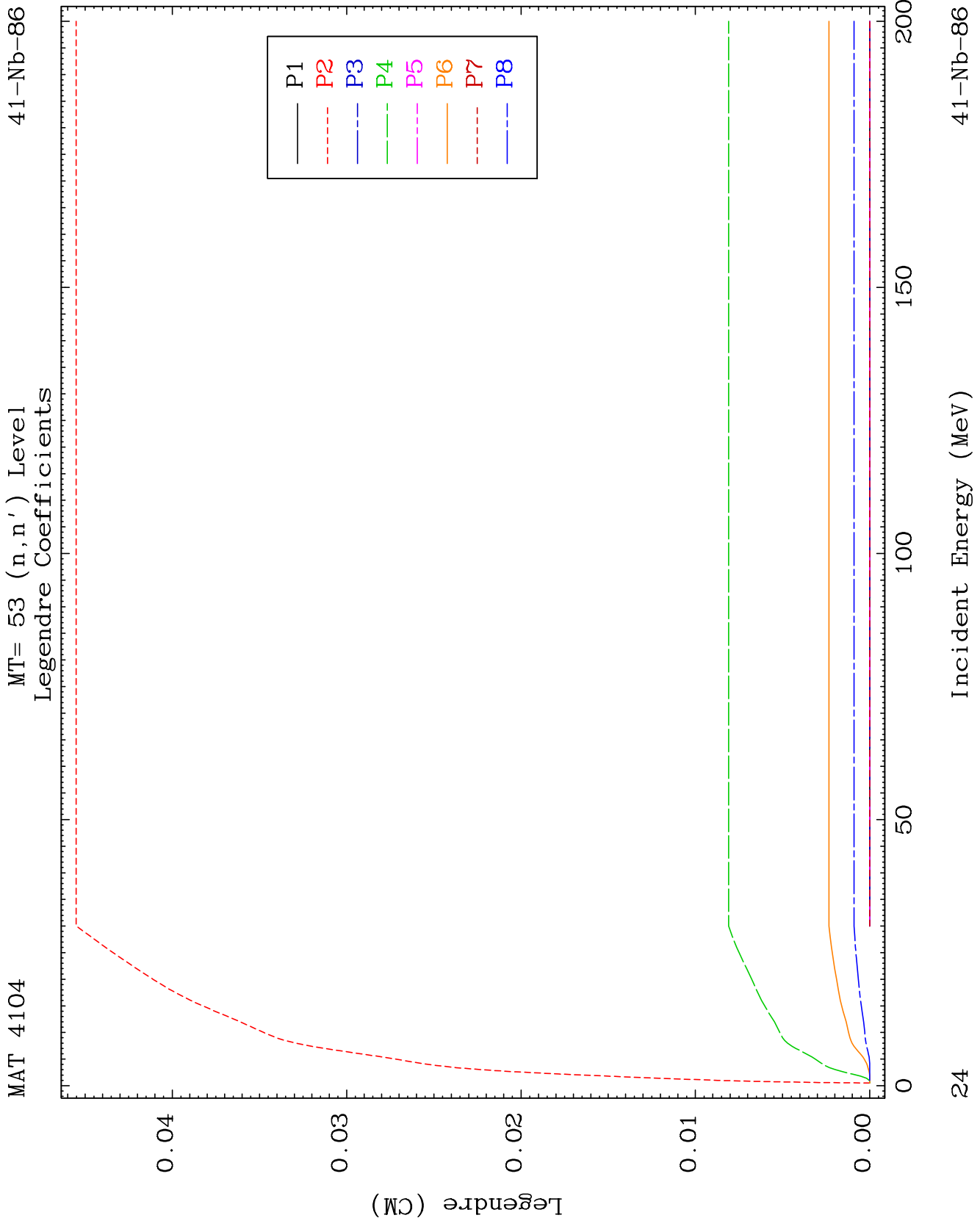
MAT 4104

MT= 52 (n,n') Level
Legendre Coefficients

41-Nb-86



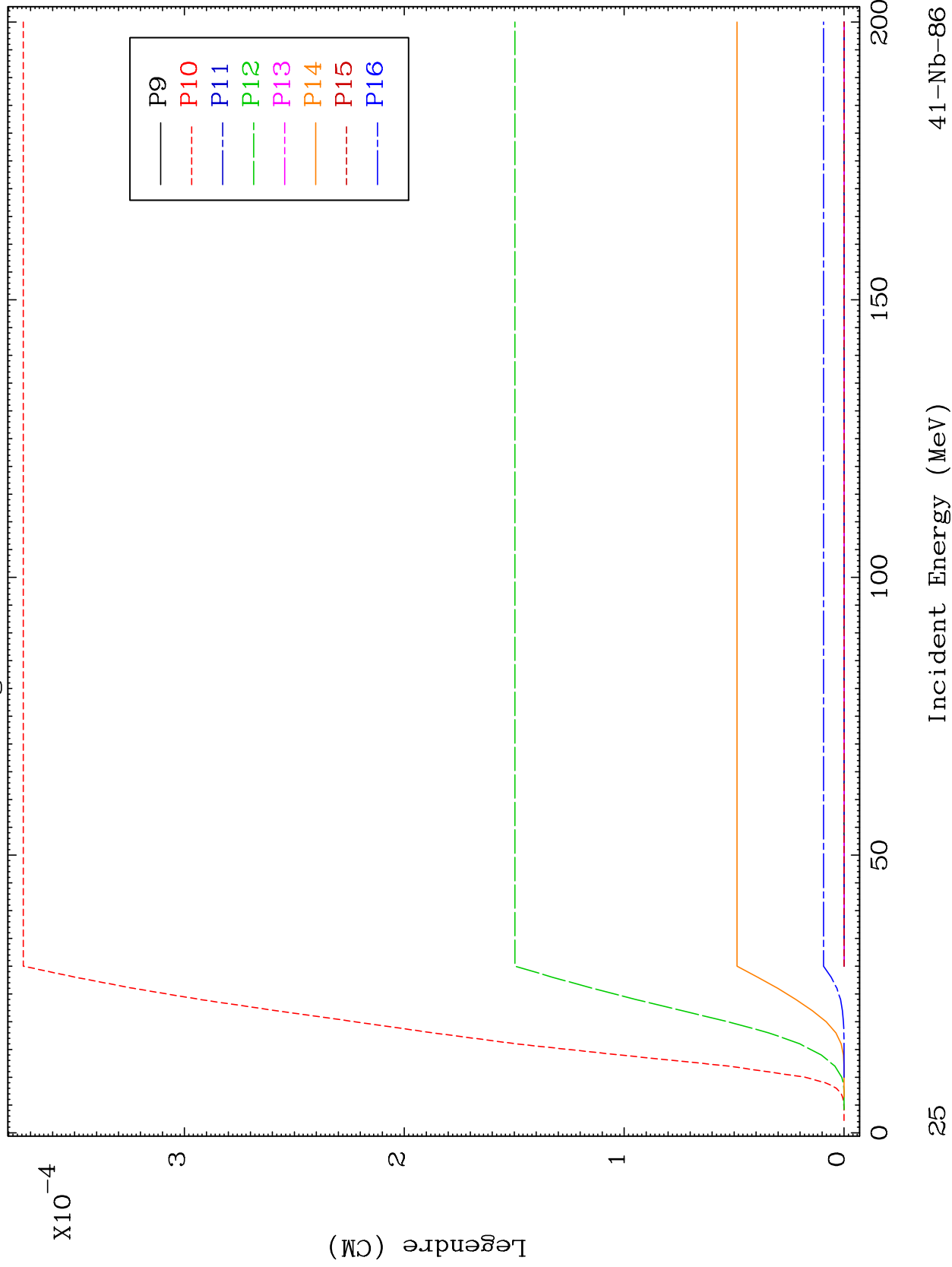
23



MAT 4104

MT= 53 (n,n') Level
Legendre Coefficients

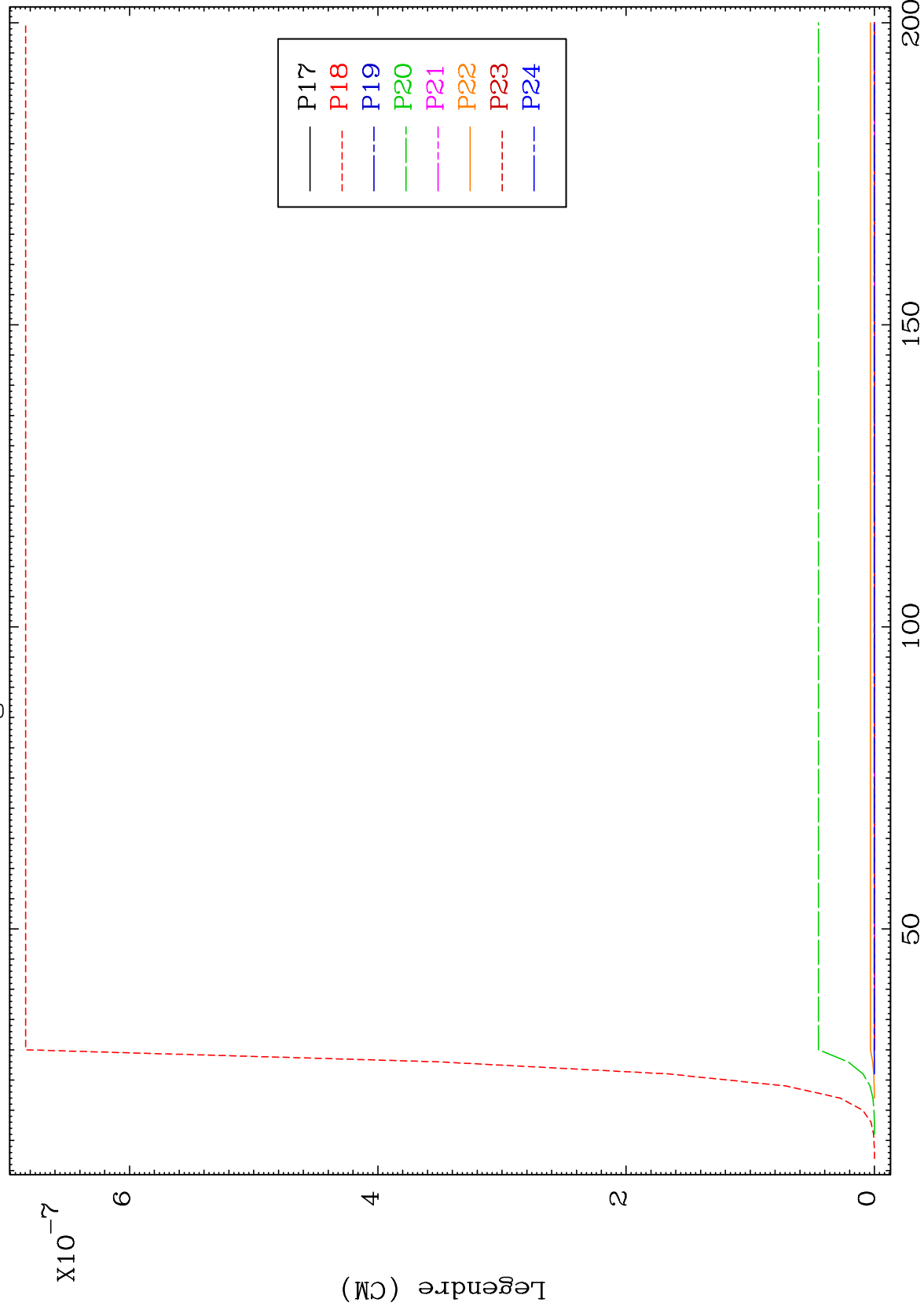
41-Nb-86



MAT 4104

MT= 53 (n,n') Level
Legendre Coefficients

41-Nb-86



26

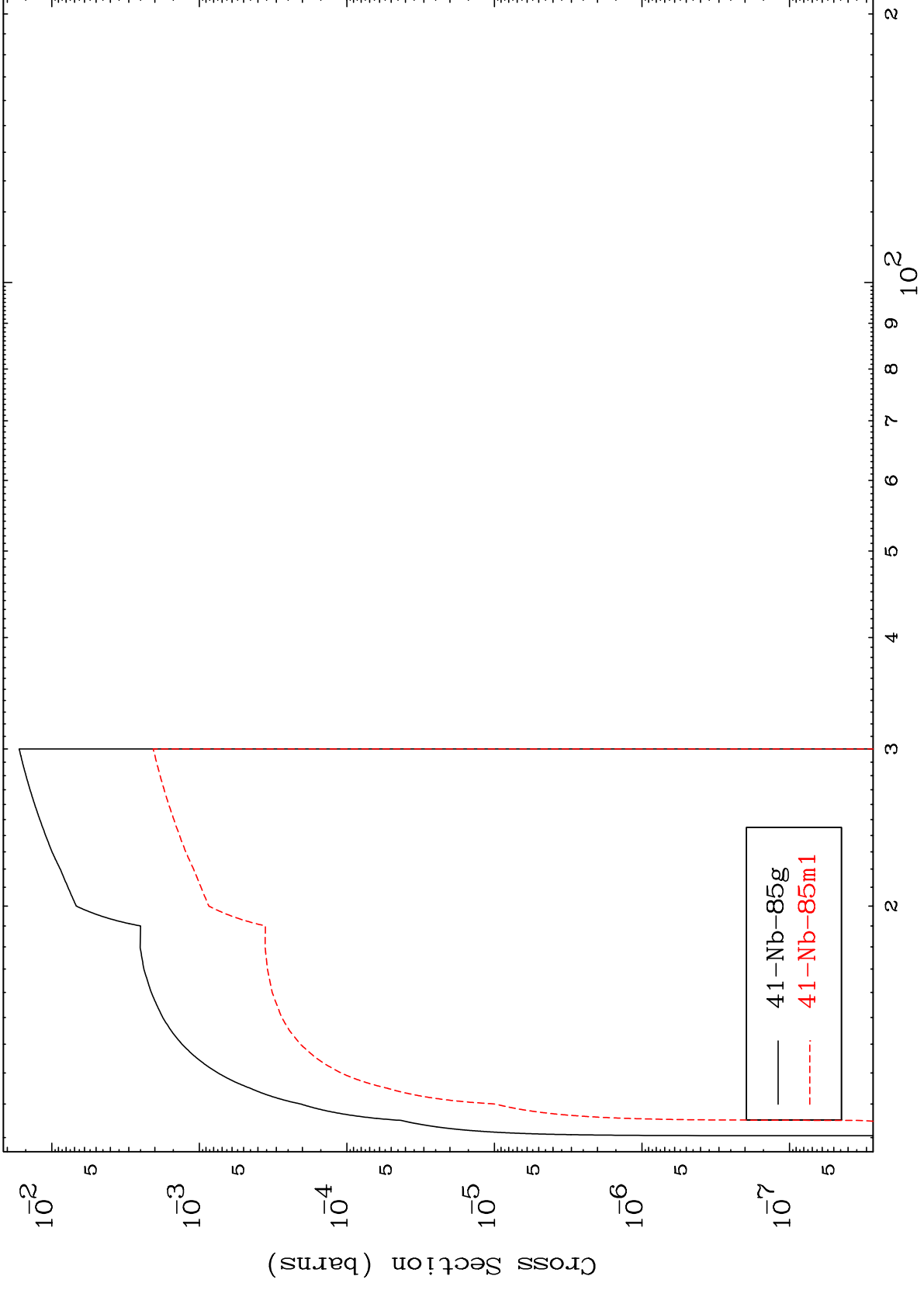
Incident Energy (MeV)

41-Nb-86

MAT 4104

41-Nb-86

(n,2n)
Radionuclide Production Cross Section



Incident Energy (MeV)

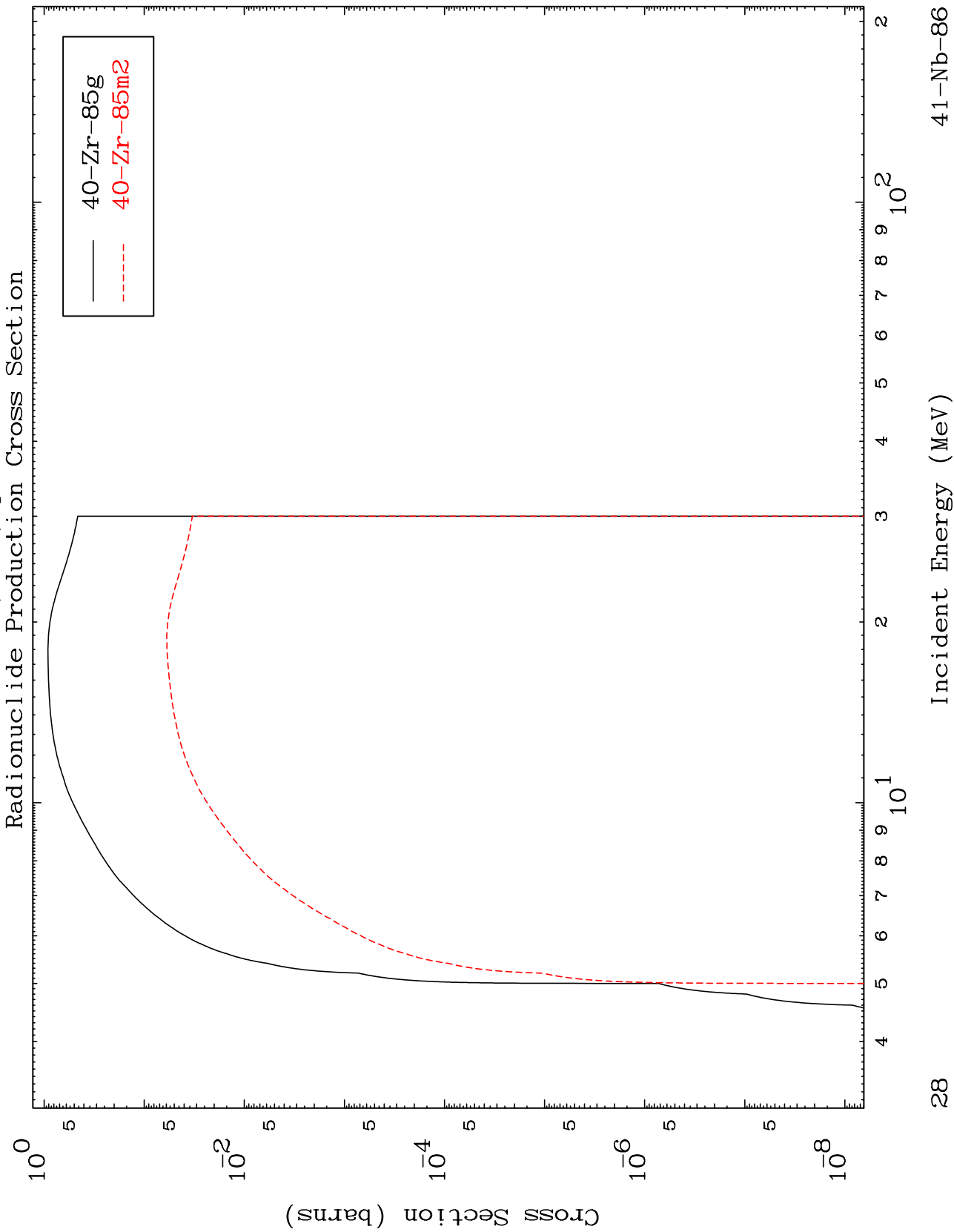
41-Nb-86

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MAT 4104

(n,n') p

41-Nb-86



28

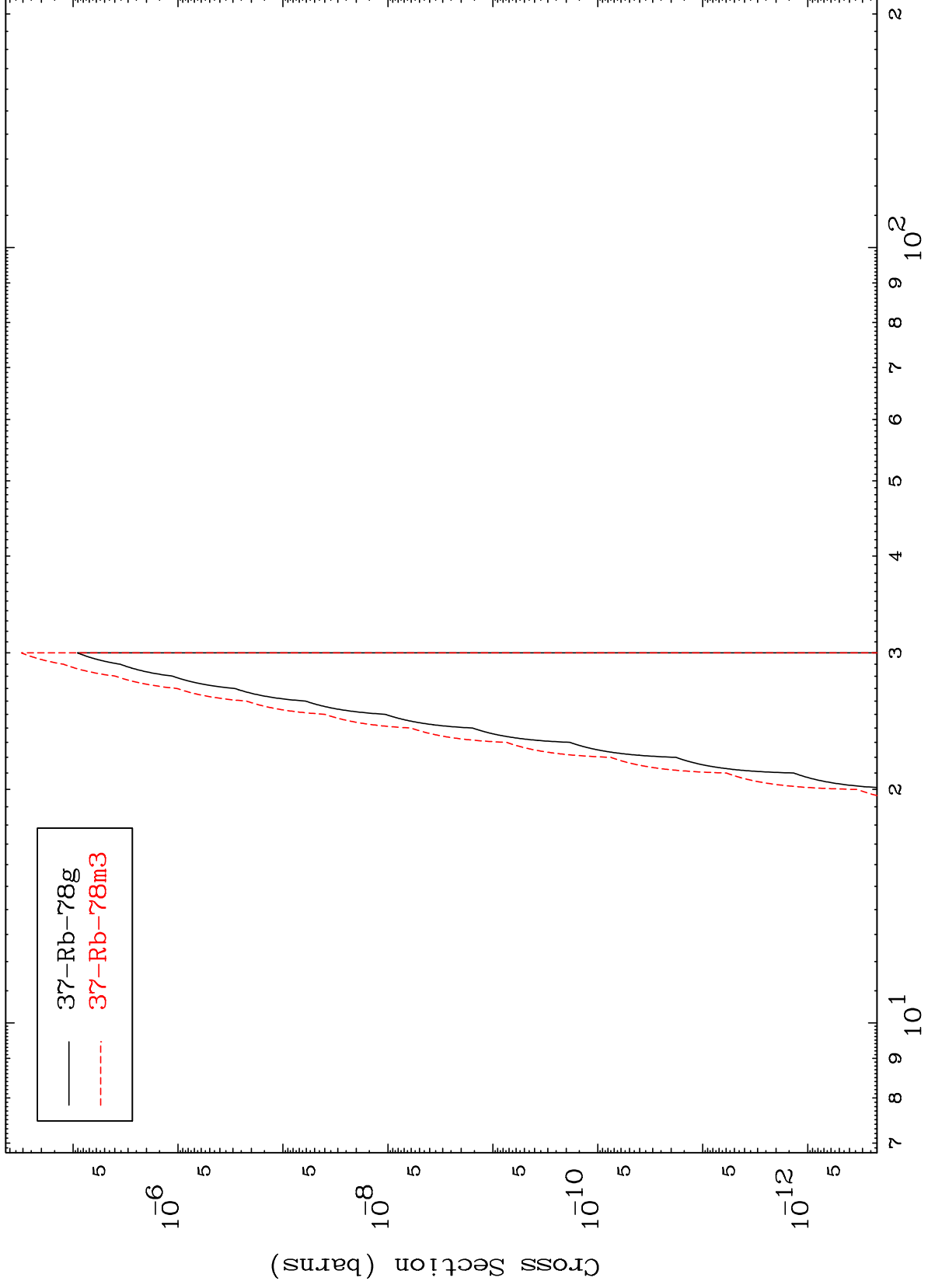
41-Nb-86

MAT 4104

(n,n') 2α

41-Nb-86

Radionuclide Production Cross Section



29

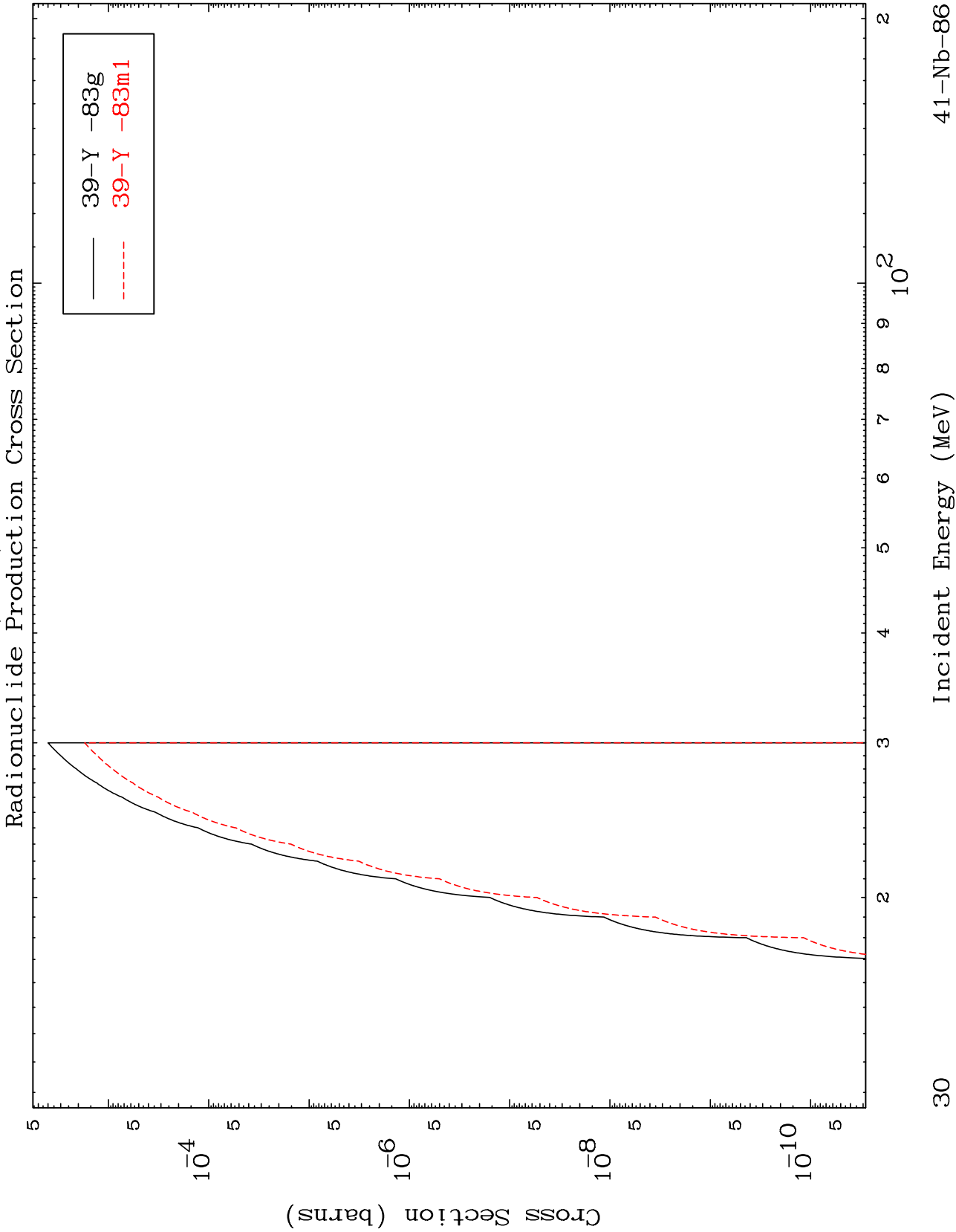
Incident Energy (MeV)

41-Nb-86

MAT 4104

(n,n') He-3

41-Nb-86



30

Incident Energy (MeV)

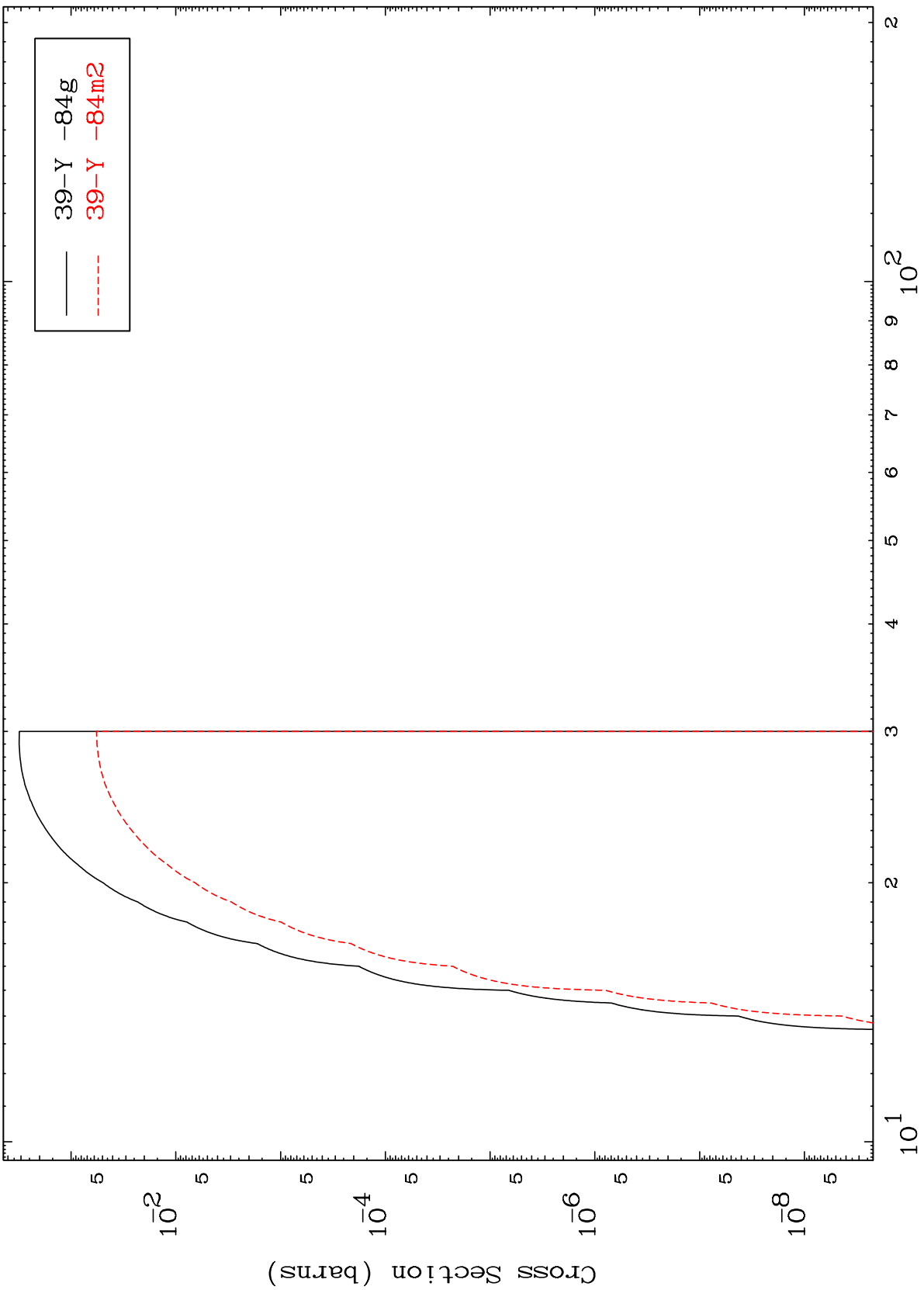
41-Nb-86

MAT 4104

(n,2n) p

41-Nb-86

Radionuclide Production Cross Section



39-Y -84g
39-Y -84m2

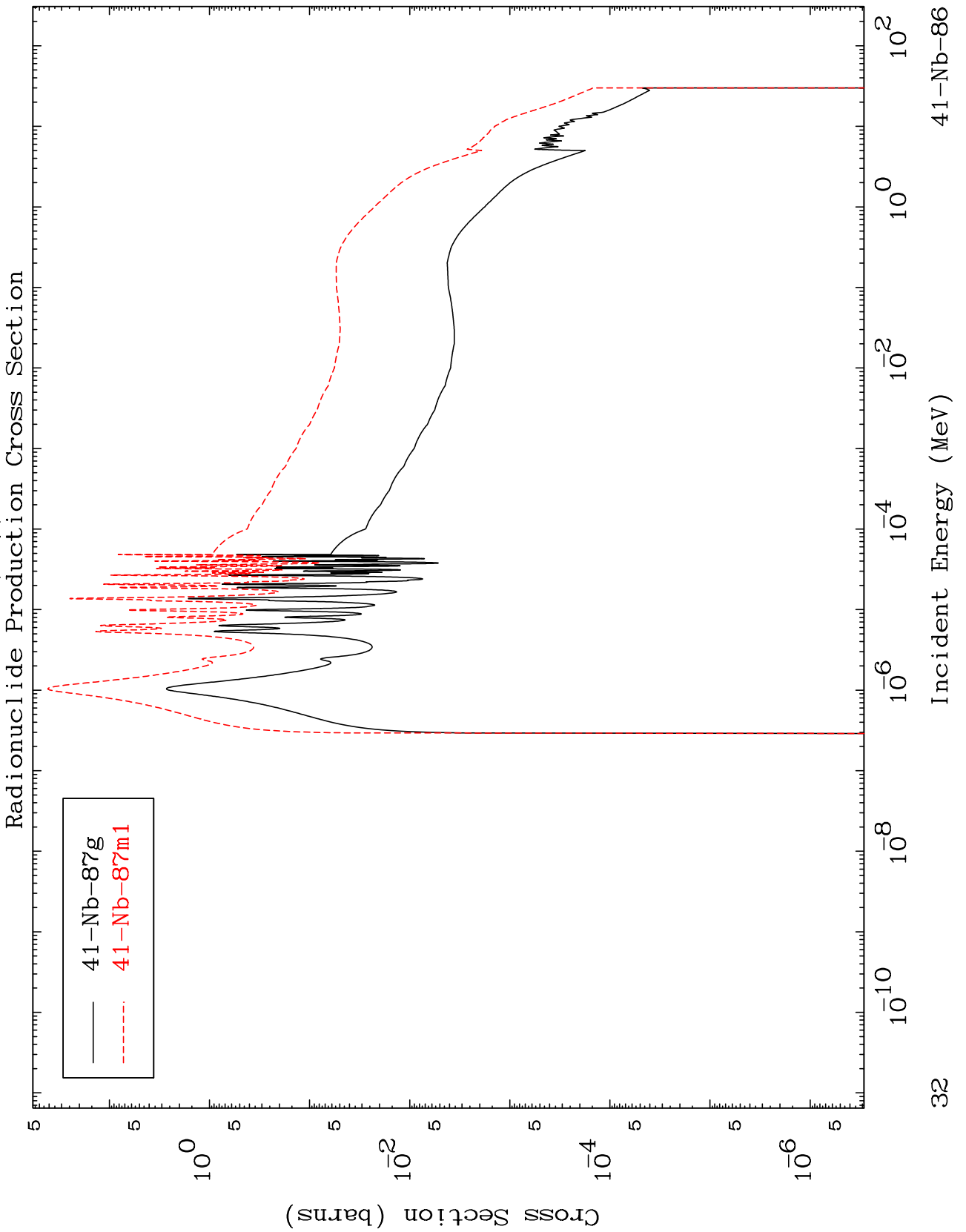
31

Incident Energy (MeV)

41-Nb-86

MAT 4104

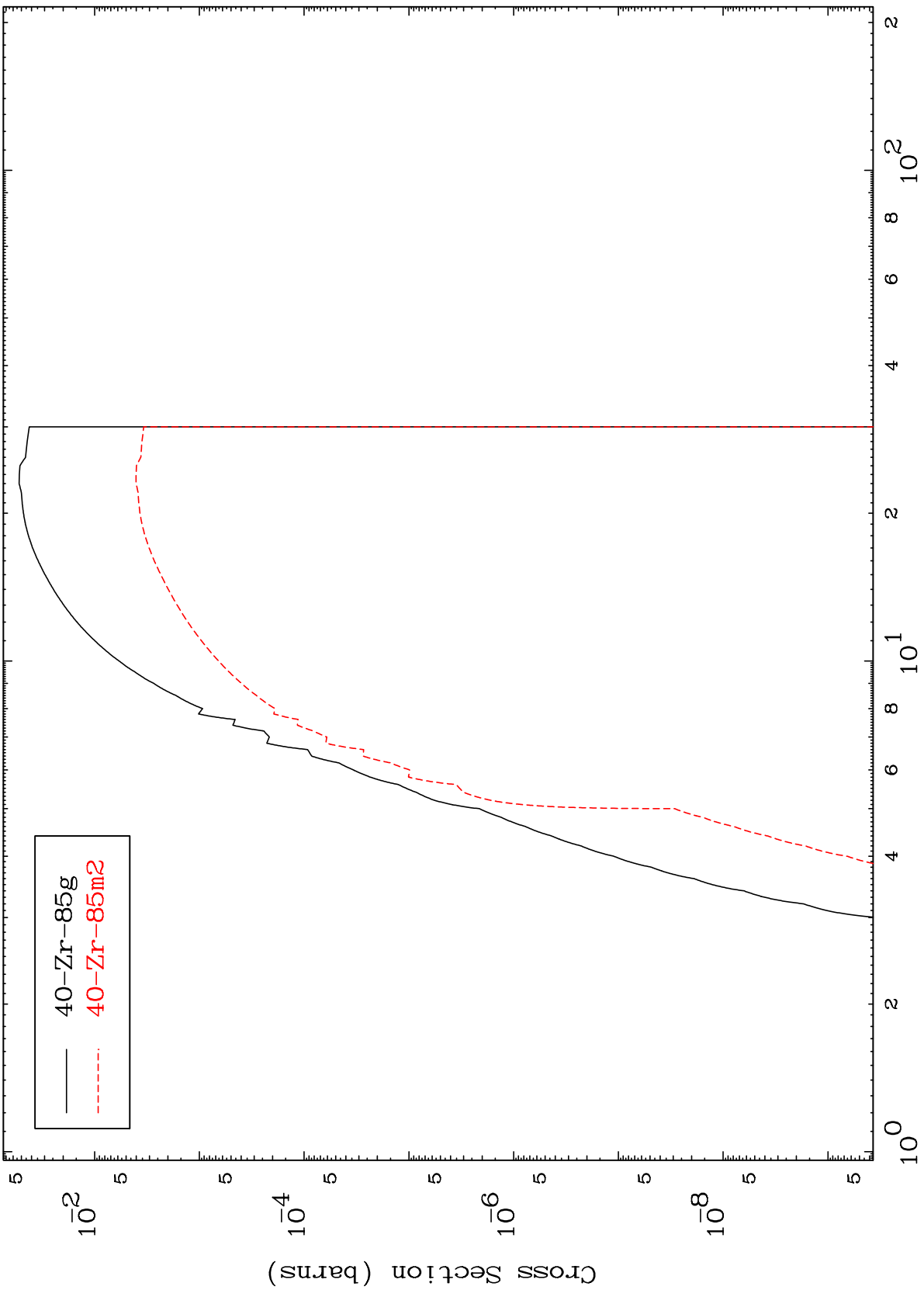
41-Nb-86



MAT 4104

41-Nb-86

(n,d)
Radionuclide Production Cross Section



— 40-Zr-85g
- - - 40-Zr-85m2

Incident Energy (MeV)

41-Nb-86

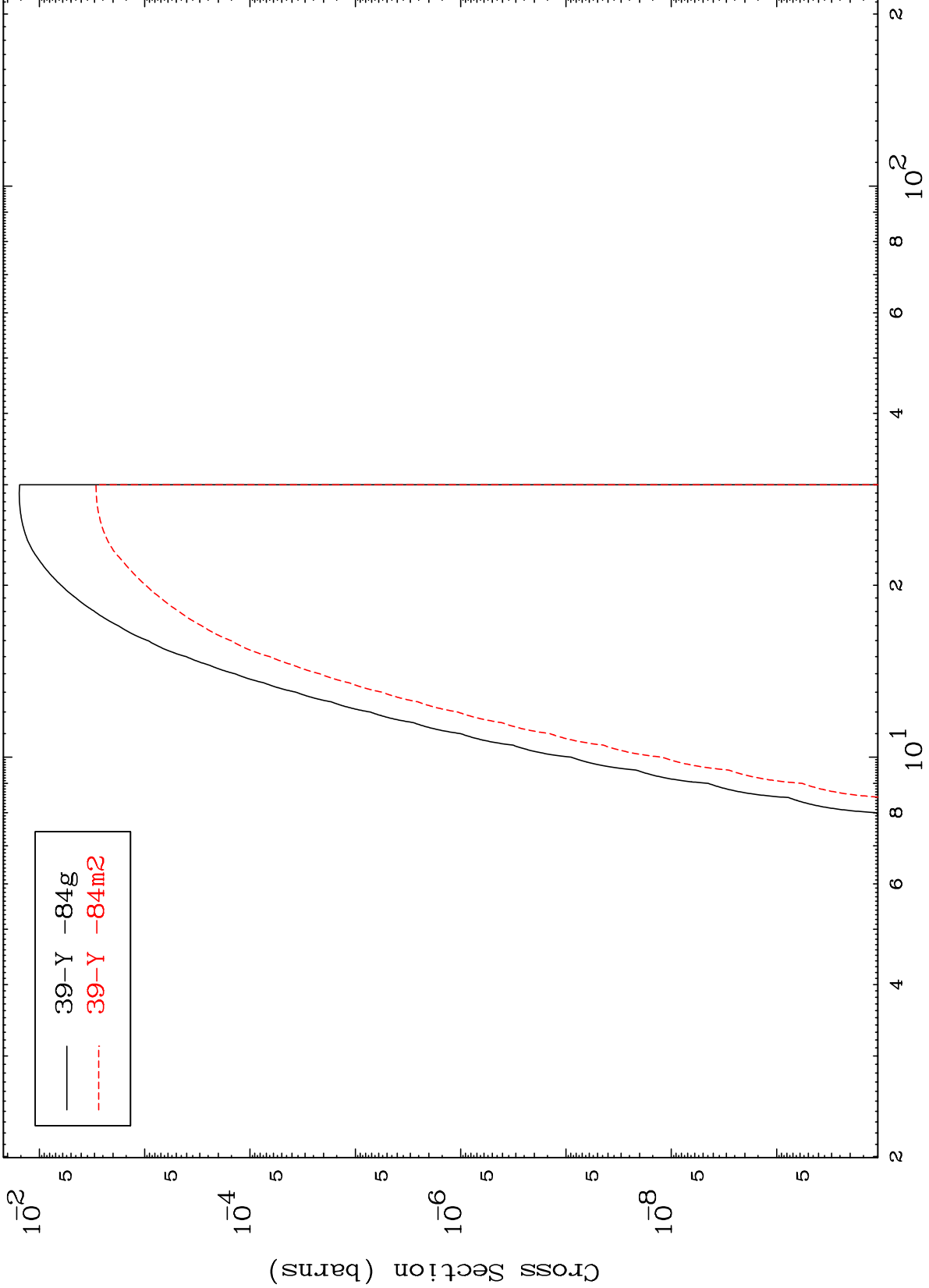
33

MAT 4104

(n,He-3)

41-Nb-86

Radionuclide Production Cross Section



— 39-Y -84g
- - - 39-Y -84m2

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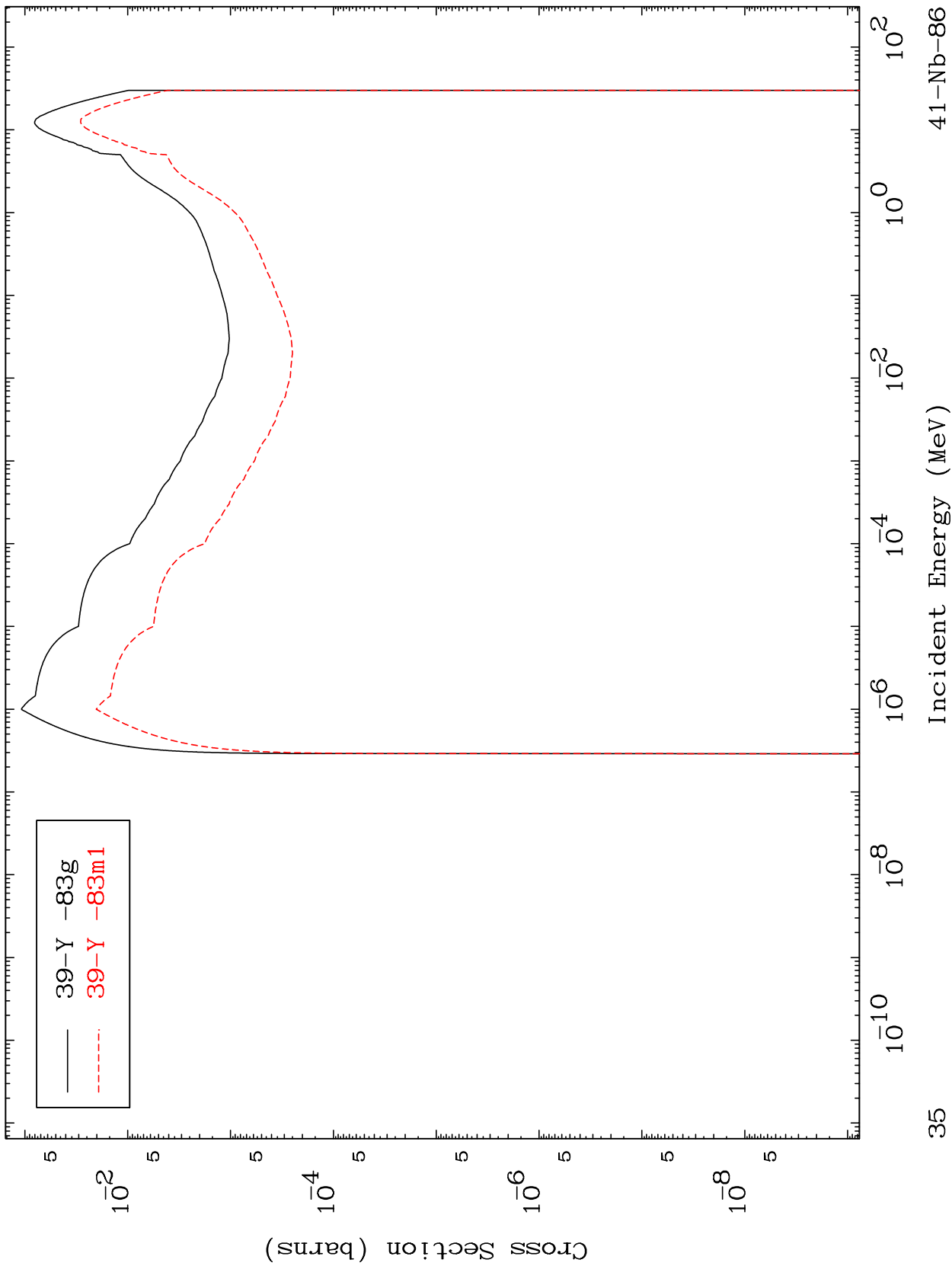
Incident Energy (MeV)

41-Nb-86

MAT 4104

41-Nb-86

Radionuclide Production Cross Section



35

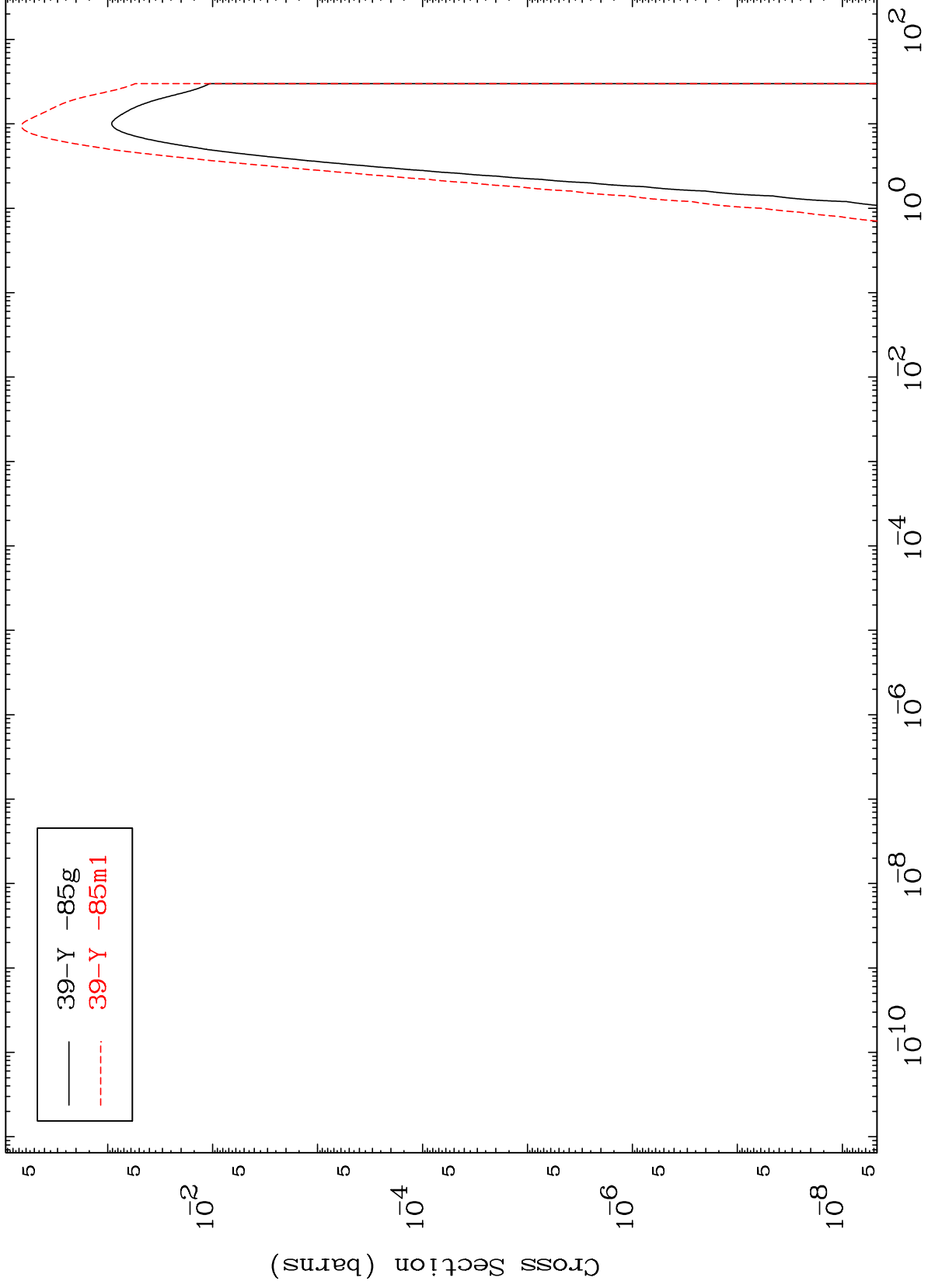
41-Nb-86

MAT 4104

(n,2p)

41-Nb-86

Radionuclide Production Cross Section



36

Incident Energy (MeV)

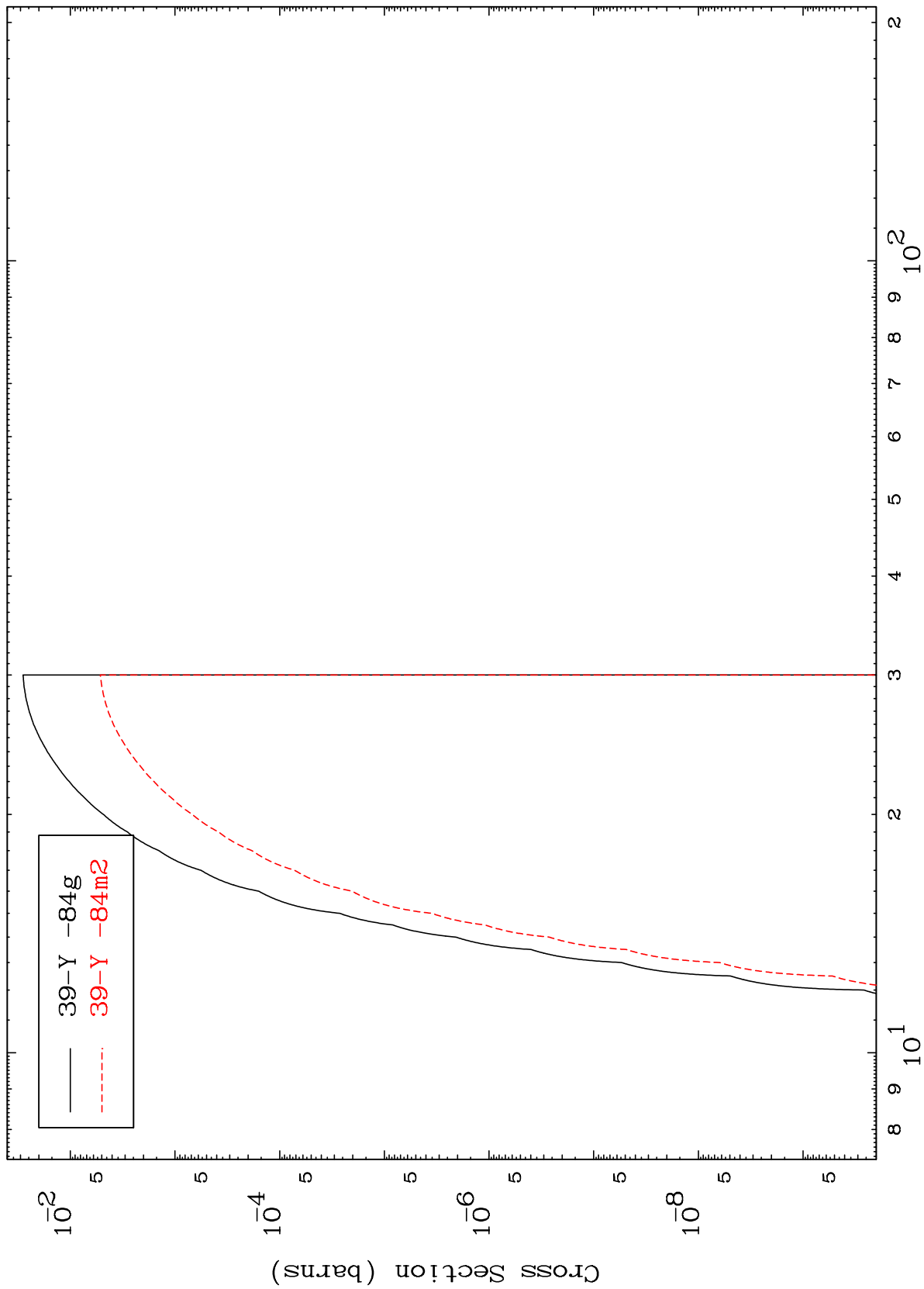
41-Nb-86

MAT 4104

(n,p) d

41-Nb-86

Radionuclide Production Cross Section



37

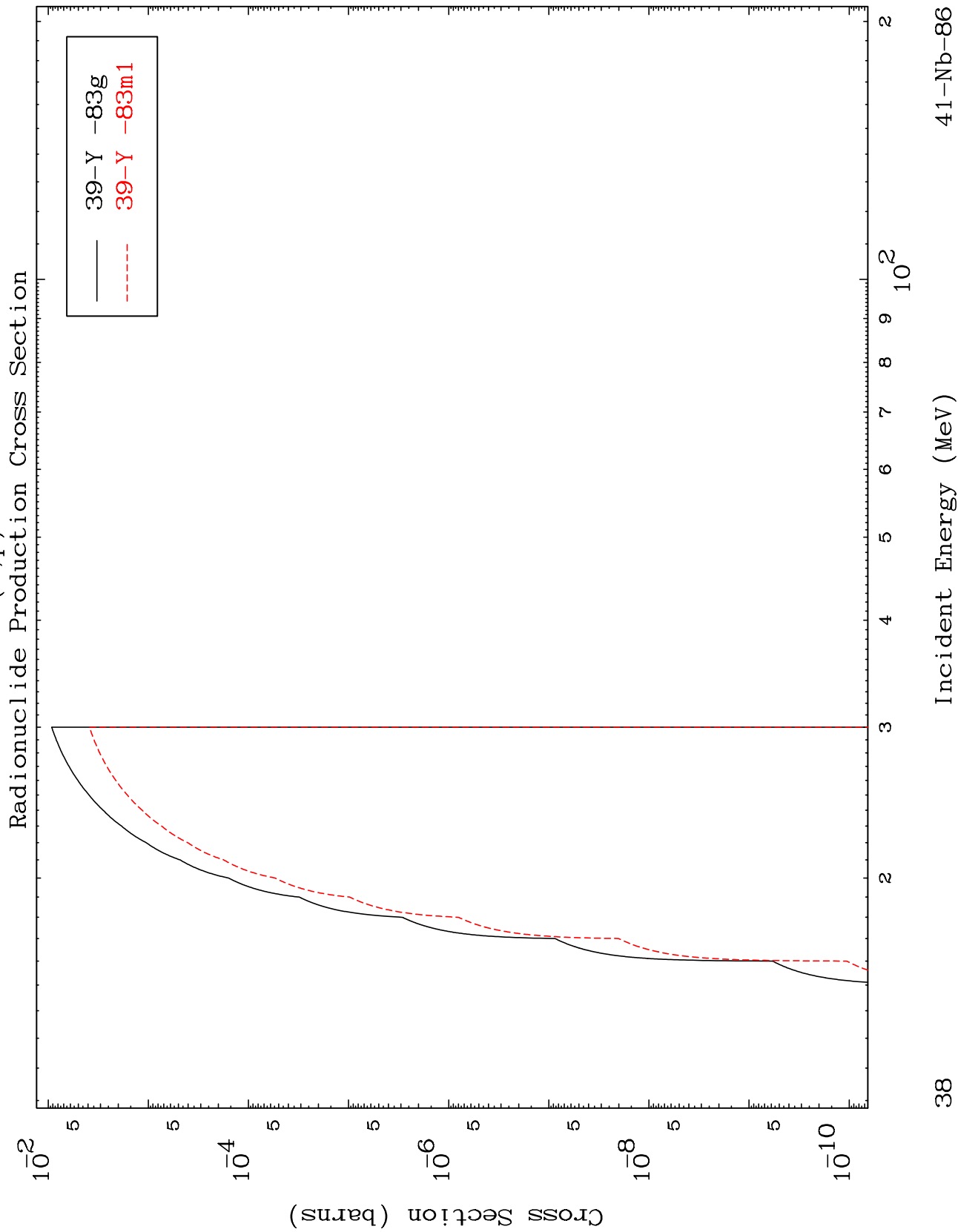
Incident Energy (MeV)

41-Nb-86

MAT 4104

(n,p) t

41-Nb-86



38

Incident Energy (MeV)

41-Nb-86