

Program EVALPLOT
(Version 2021-1)

by

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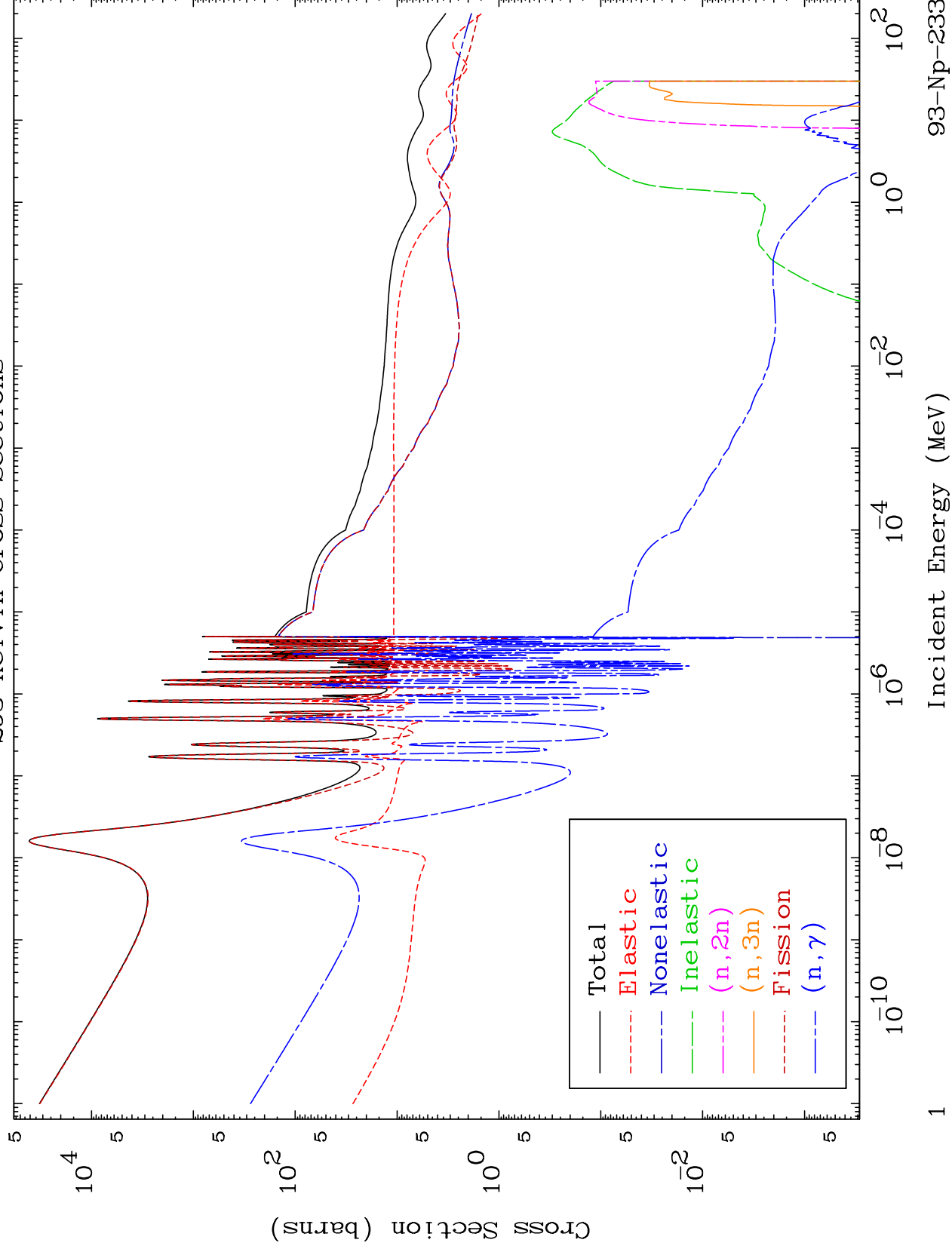
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

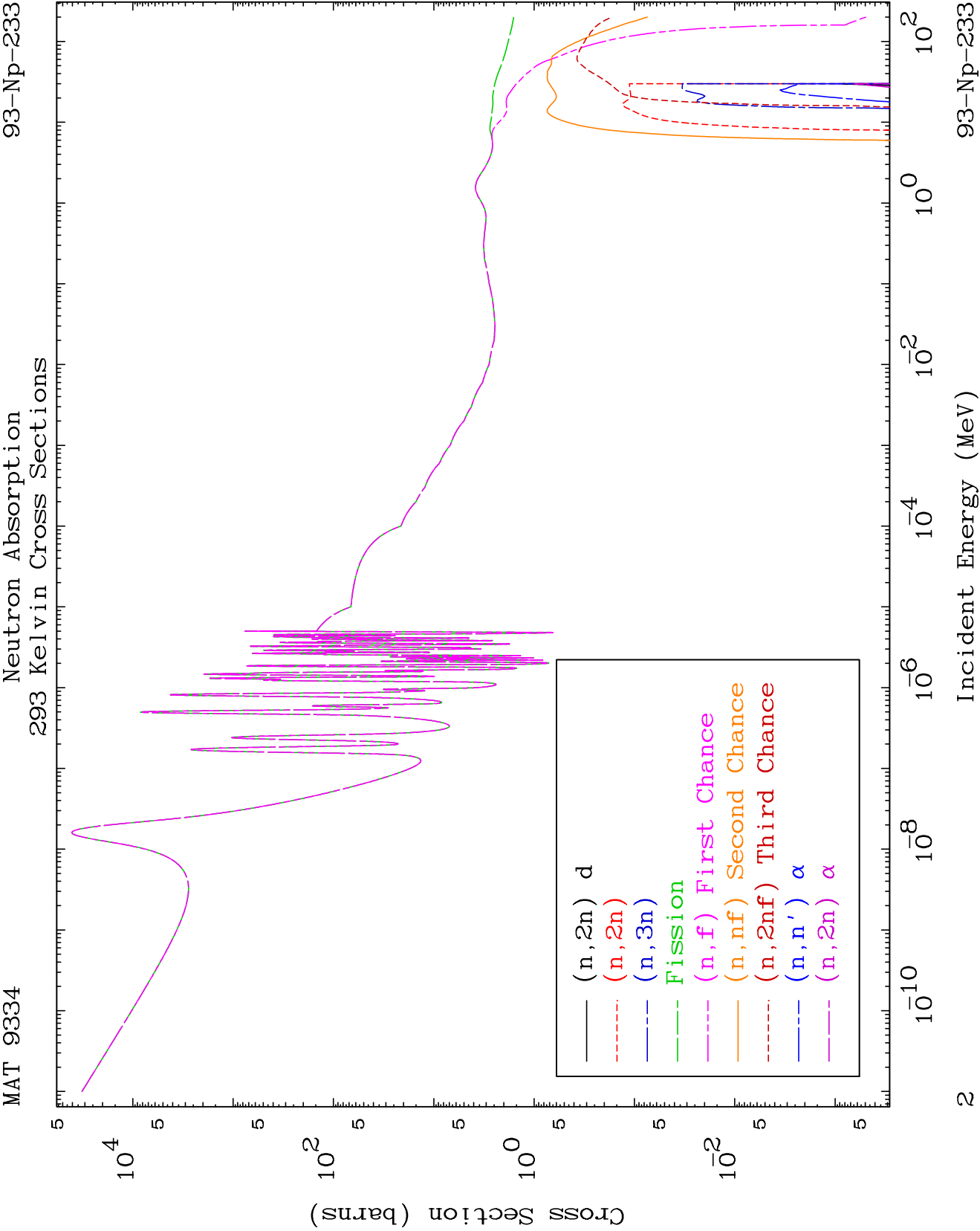
Press Mouse Button to Start

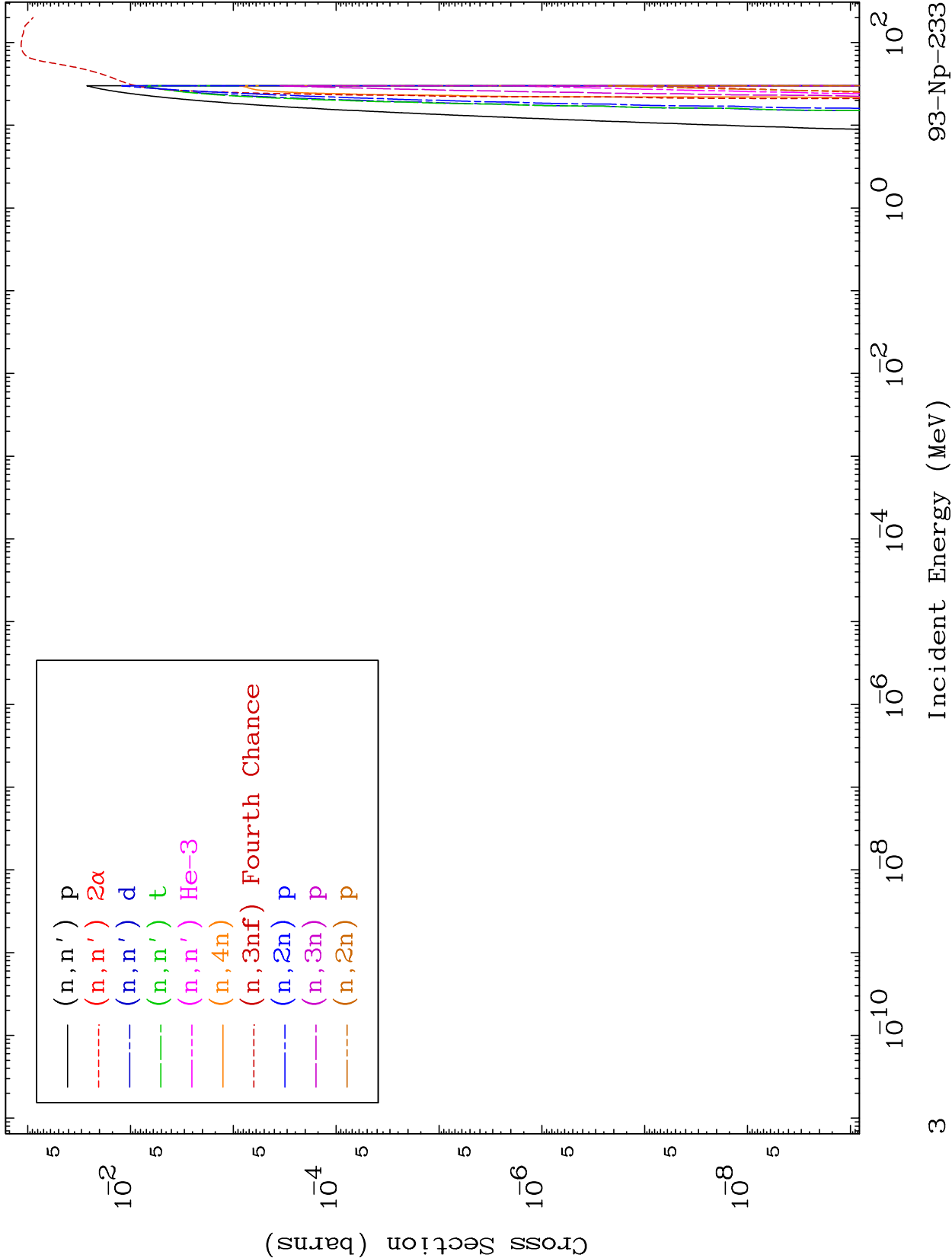
MAT 9334

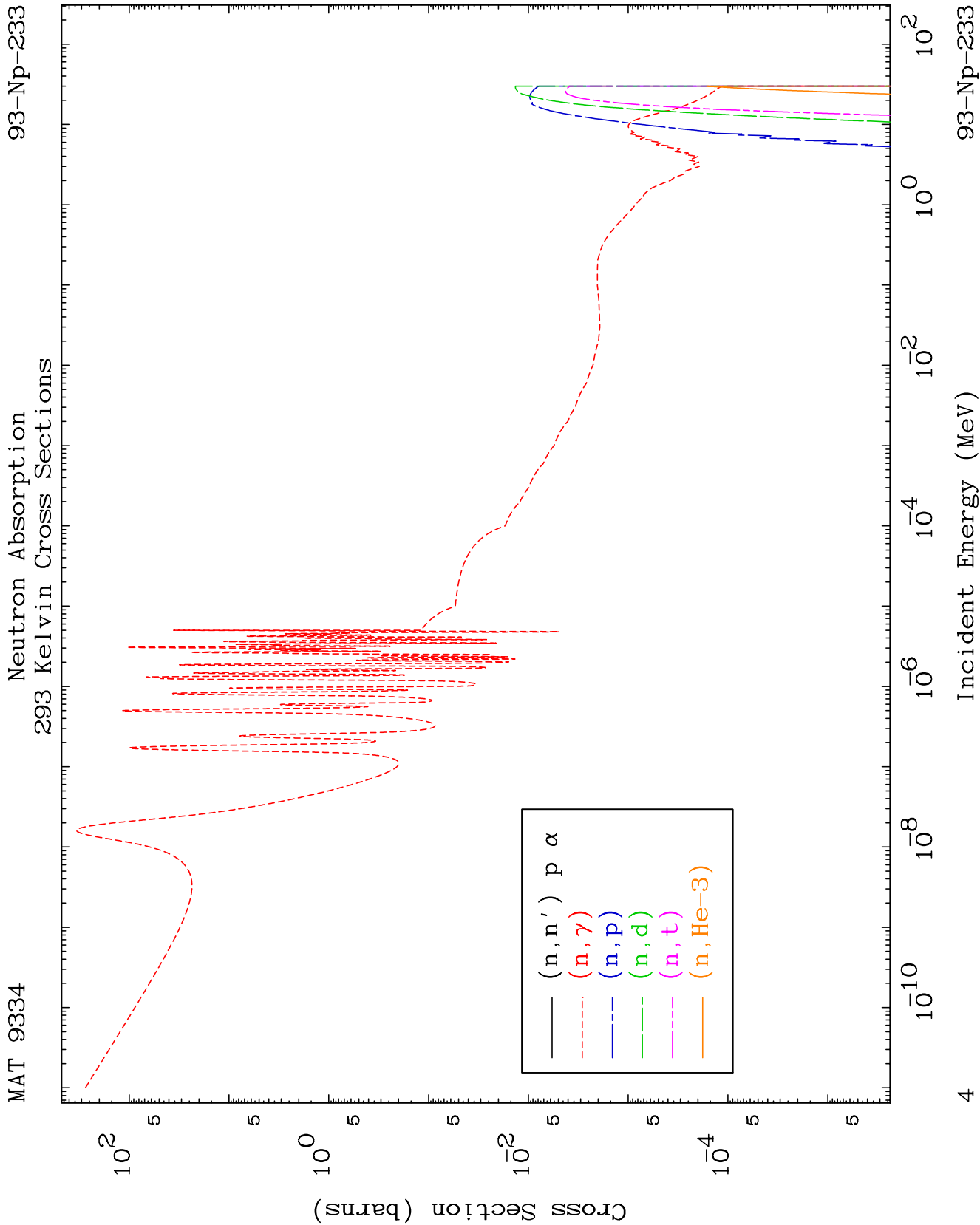
Neutron Major
293 Kelvin Cross Sections

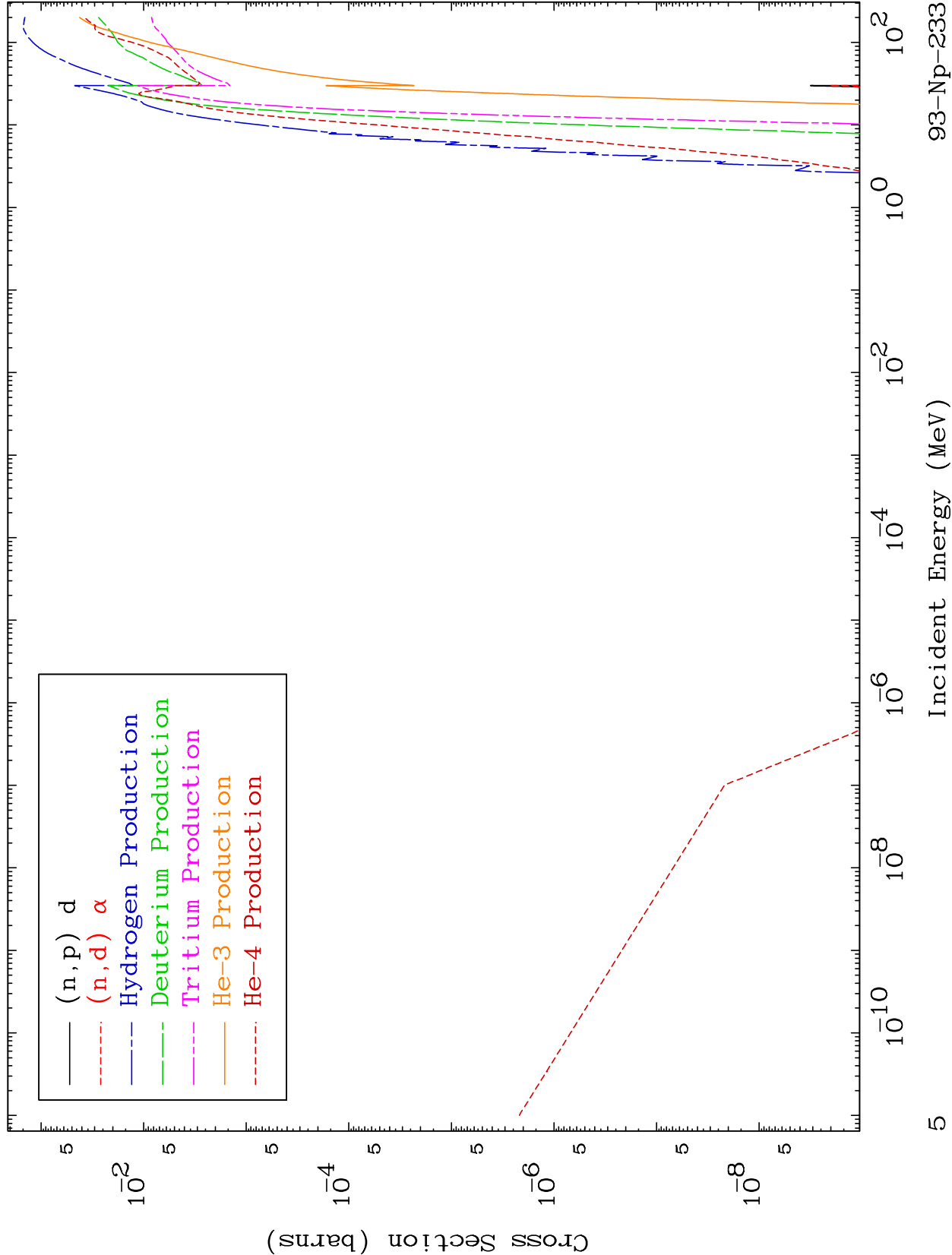
93-Np-233







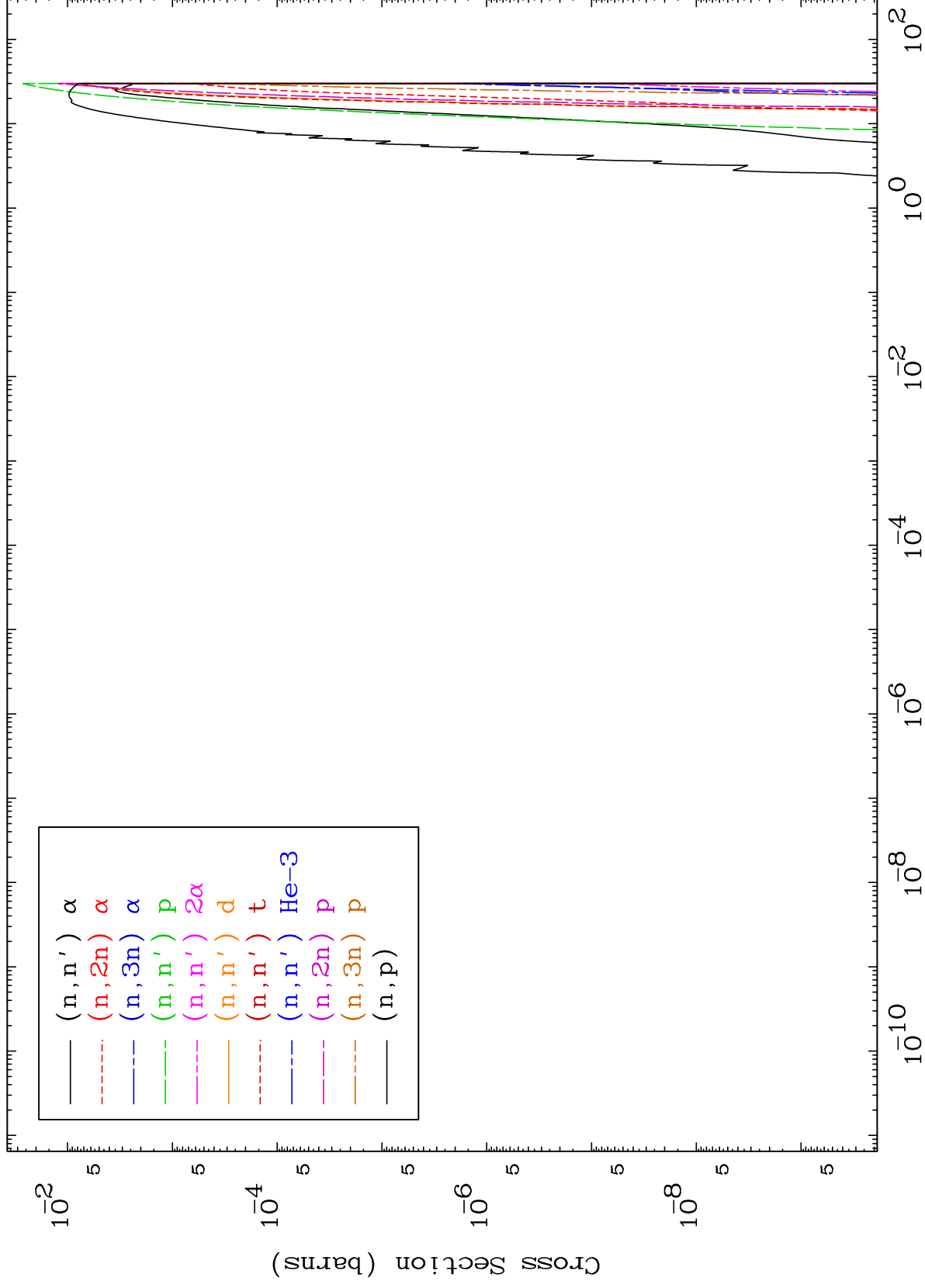


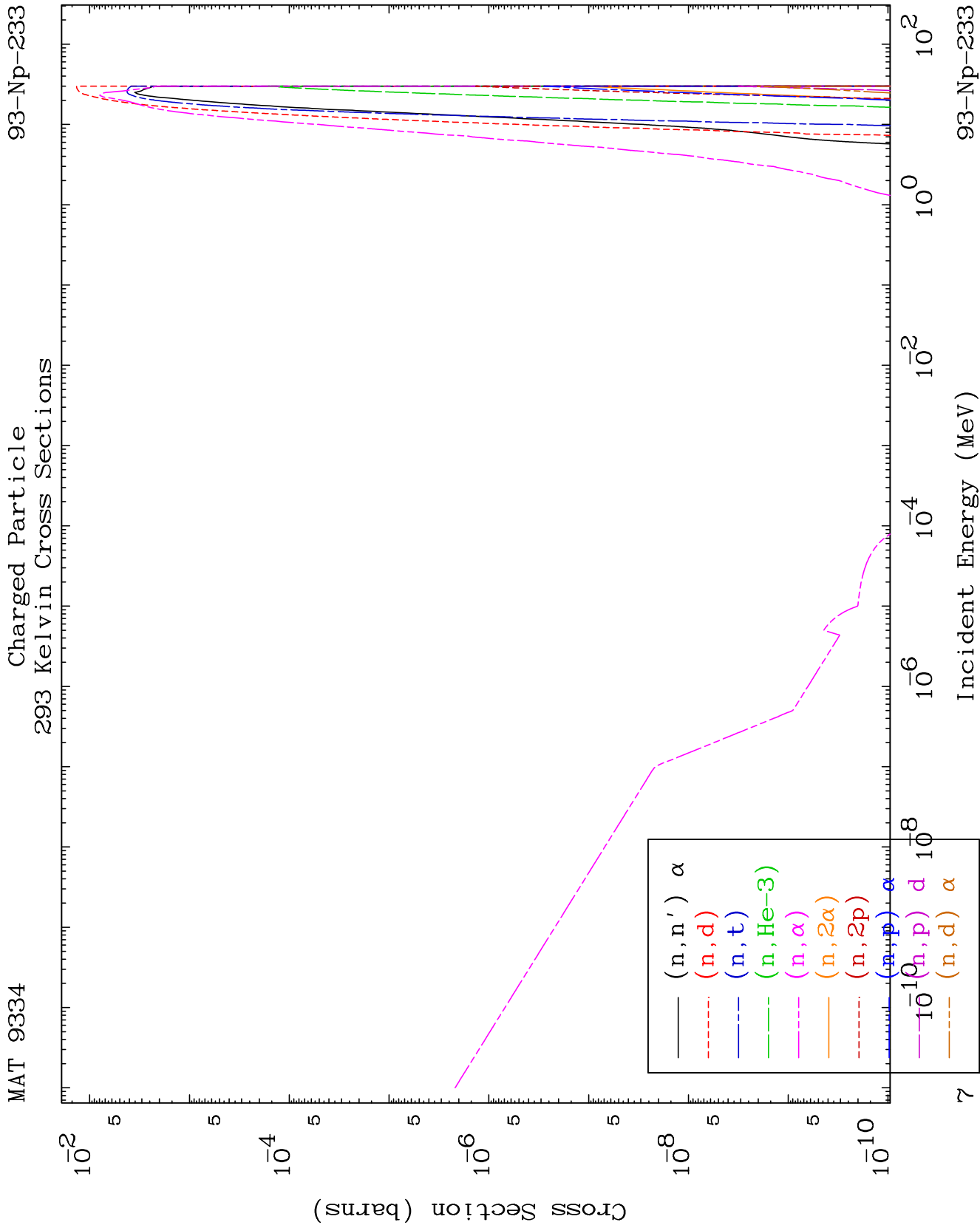


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Charged Particle
293 Kelvin Cross Sections

93-Np-233

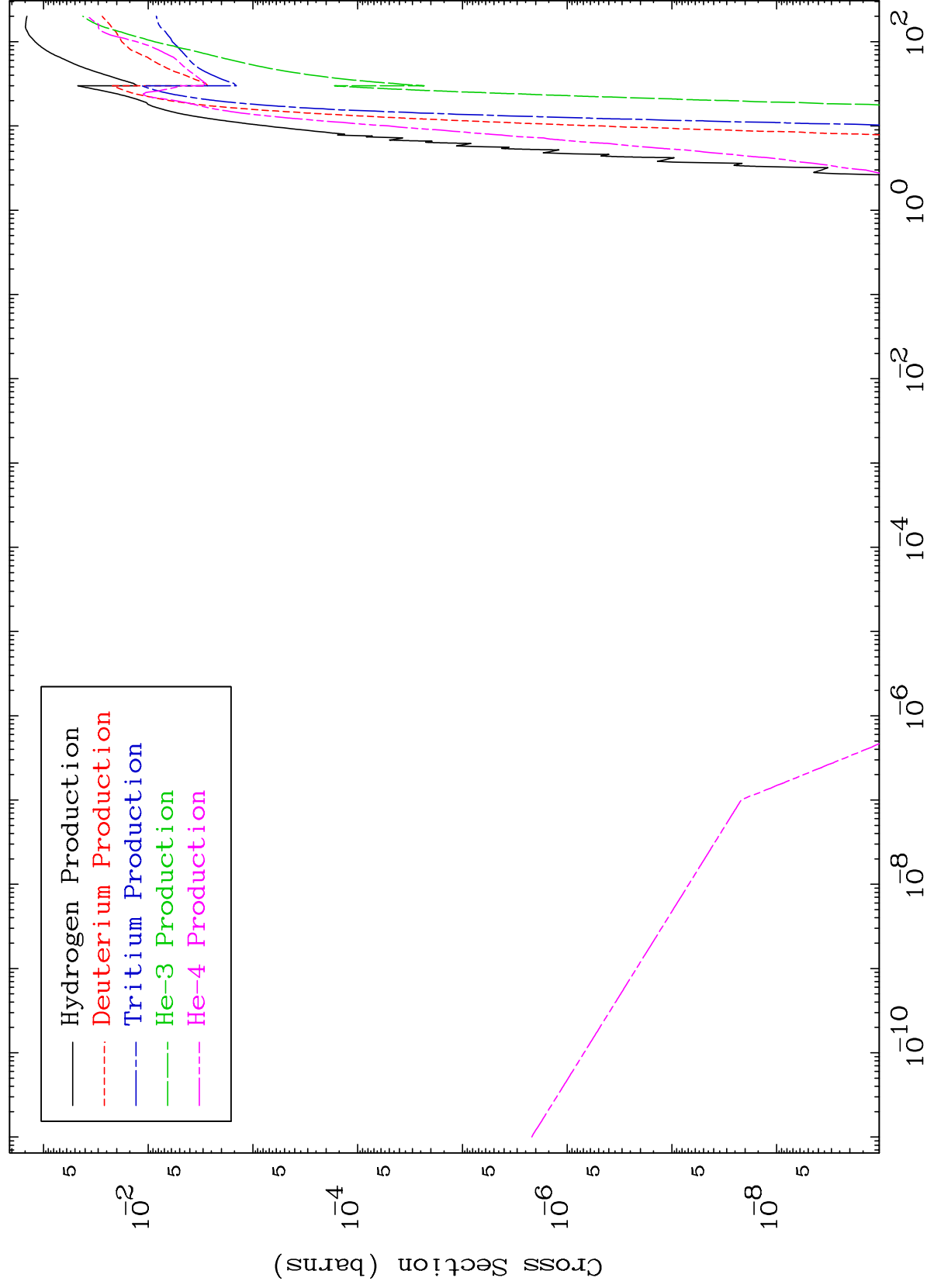


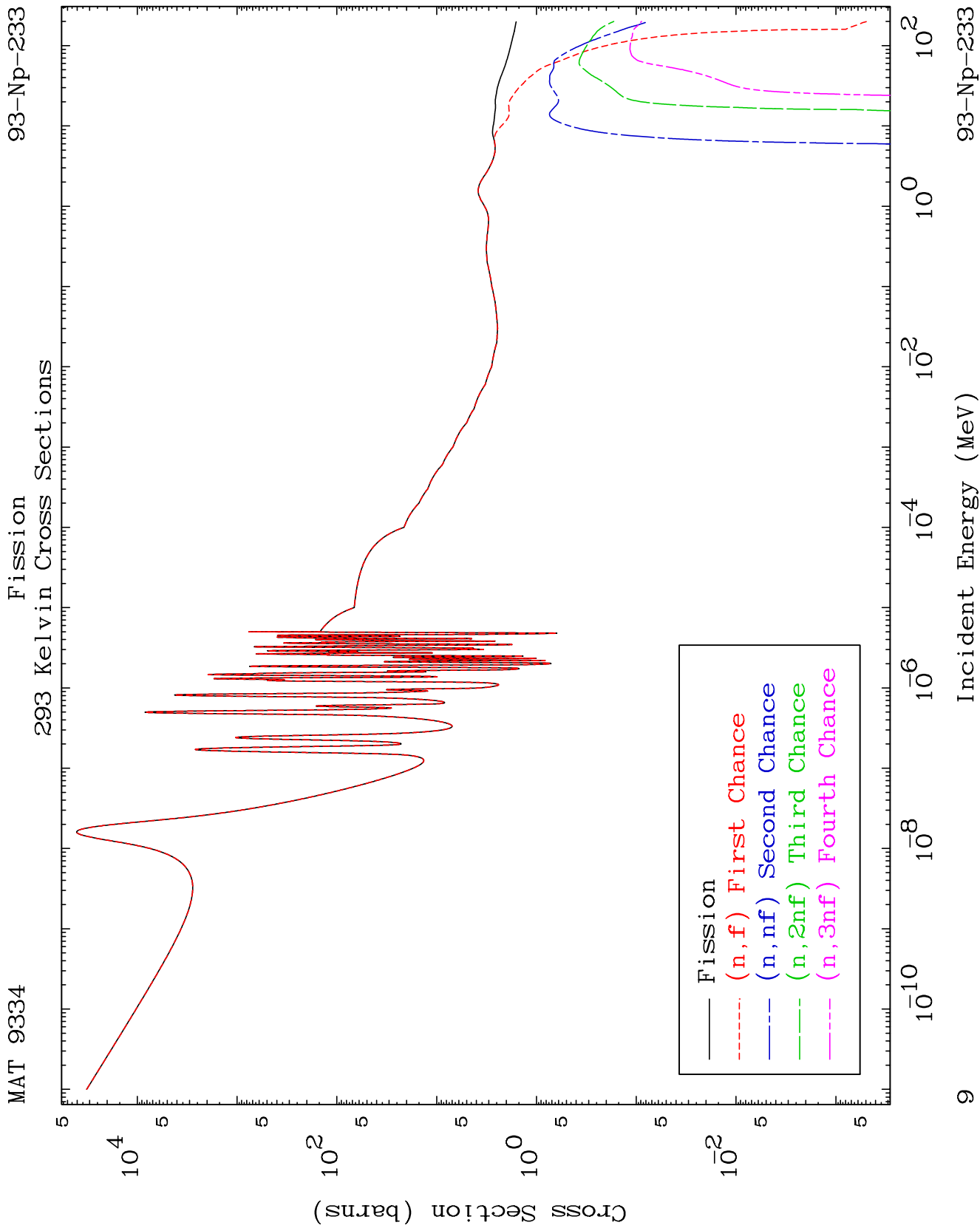


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Particle Production
293 Kelvin Cross Sections

93-Np-233

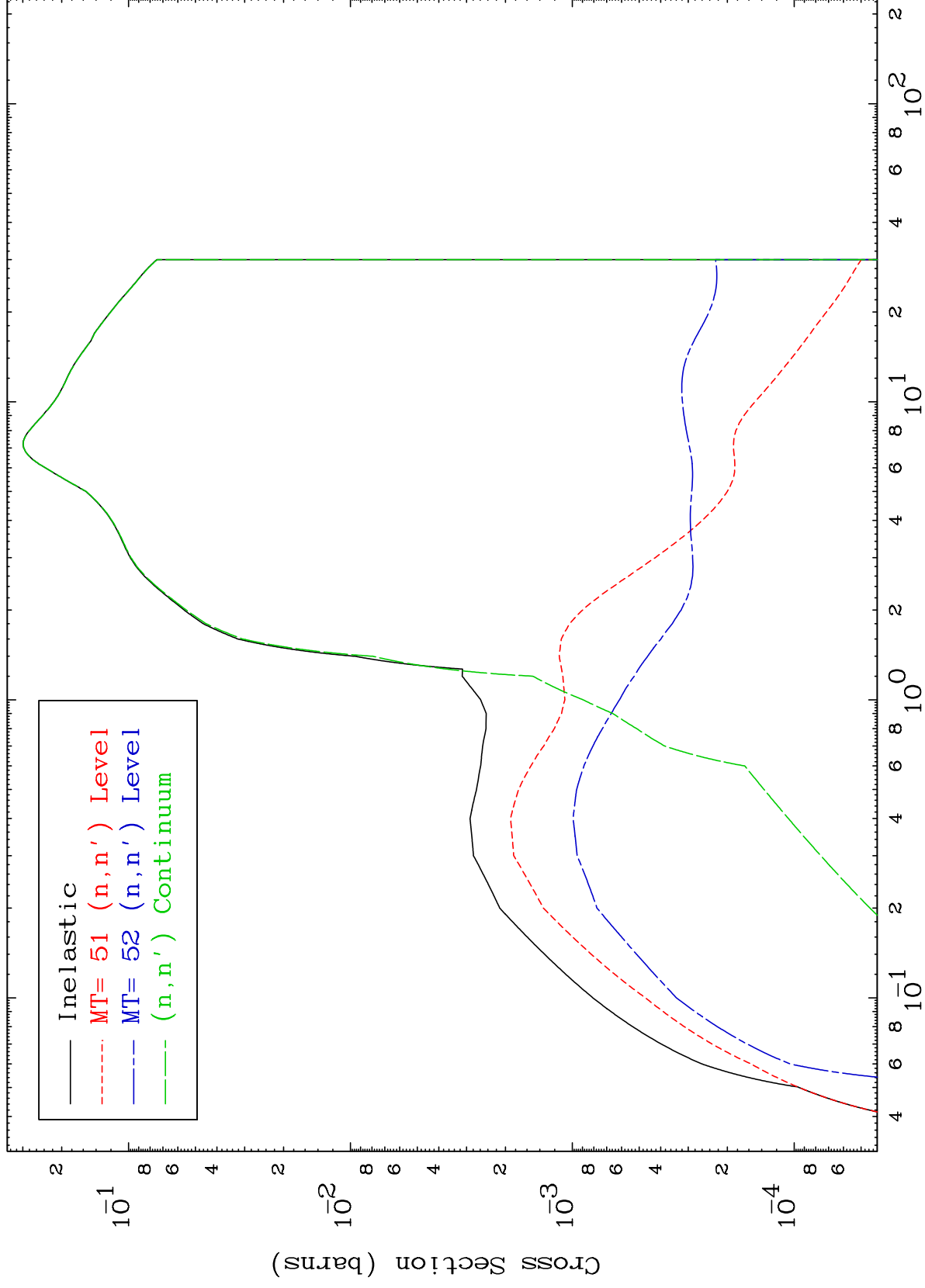




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93-Np-233

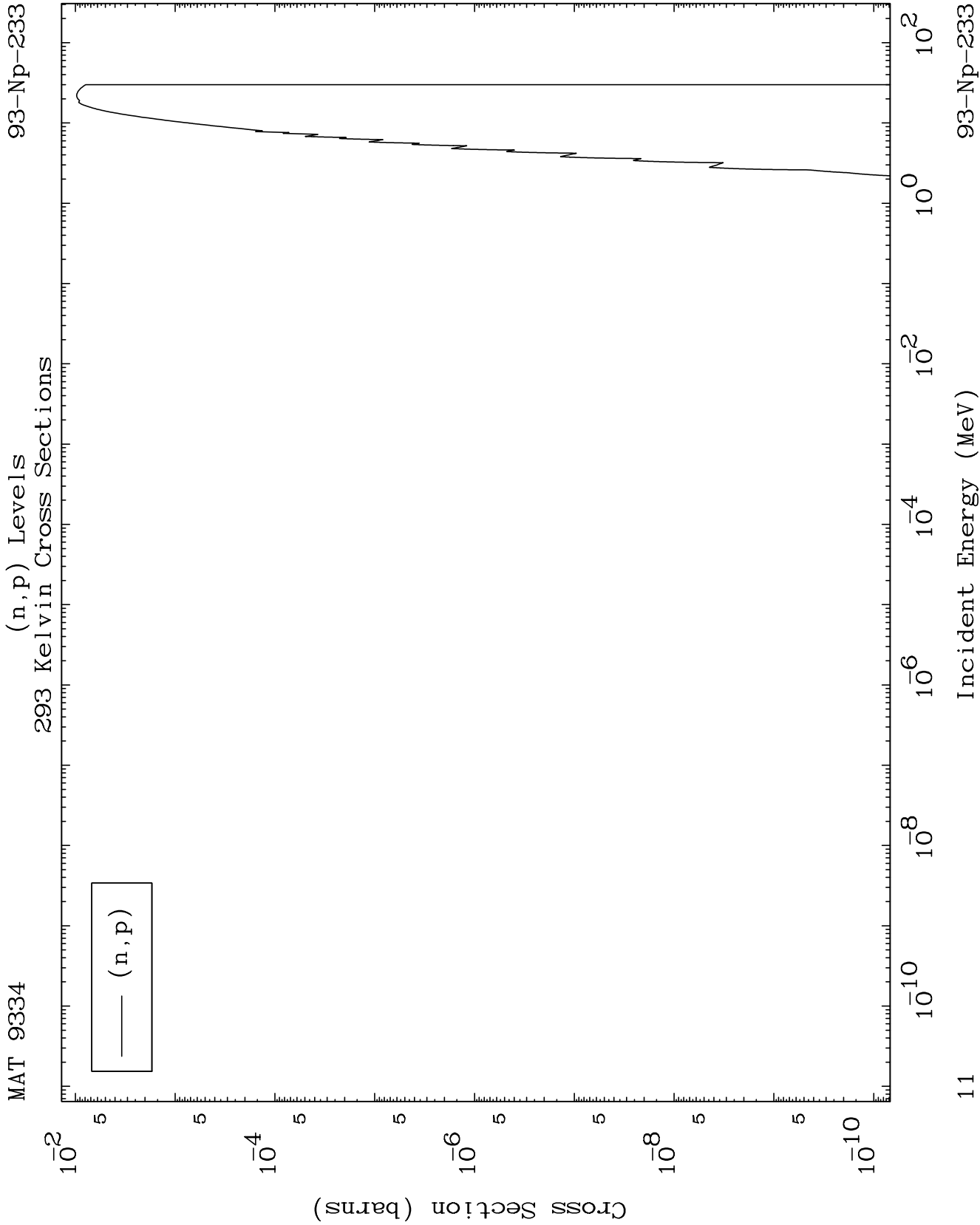
(n,n') Levels
293 Kelvin Cross Sections



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Incident Energy (MeV)

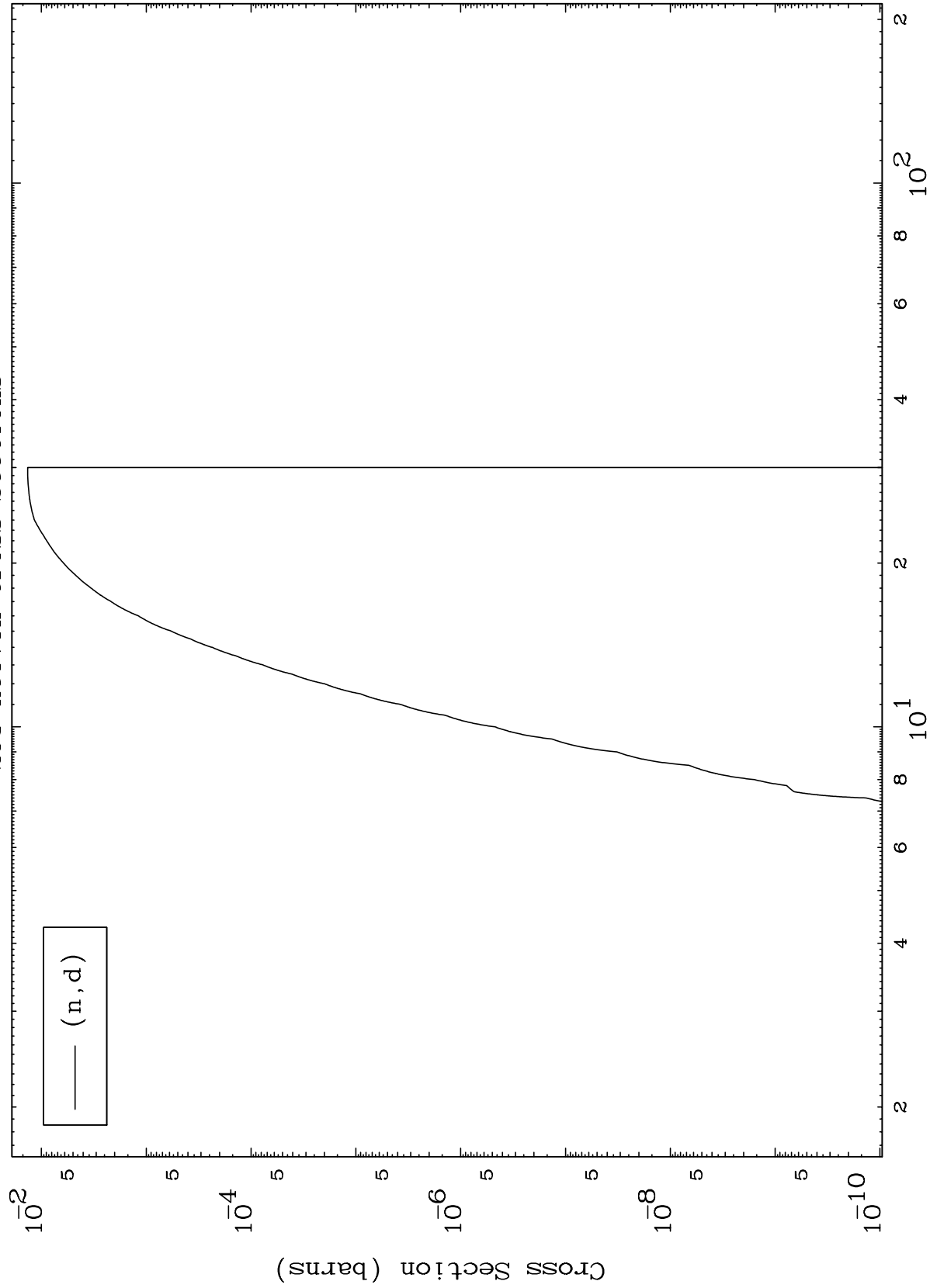
93-Np-233



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93-Np-233

(n,d) Levels
293 Kelvin Cross Sections



93-Np-233

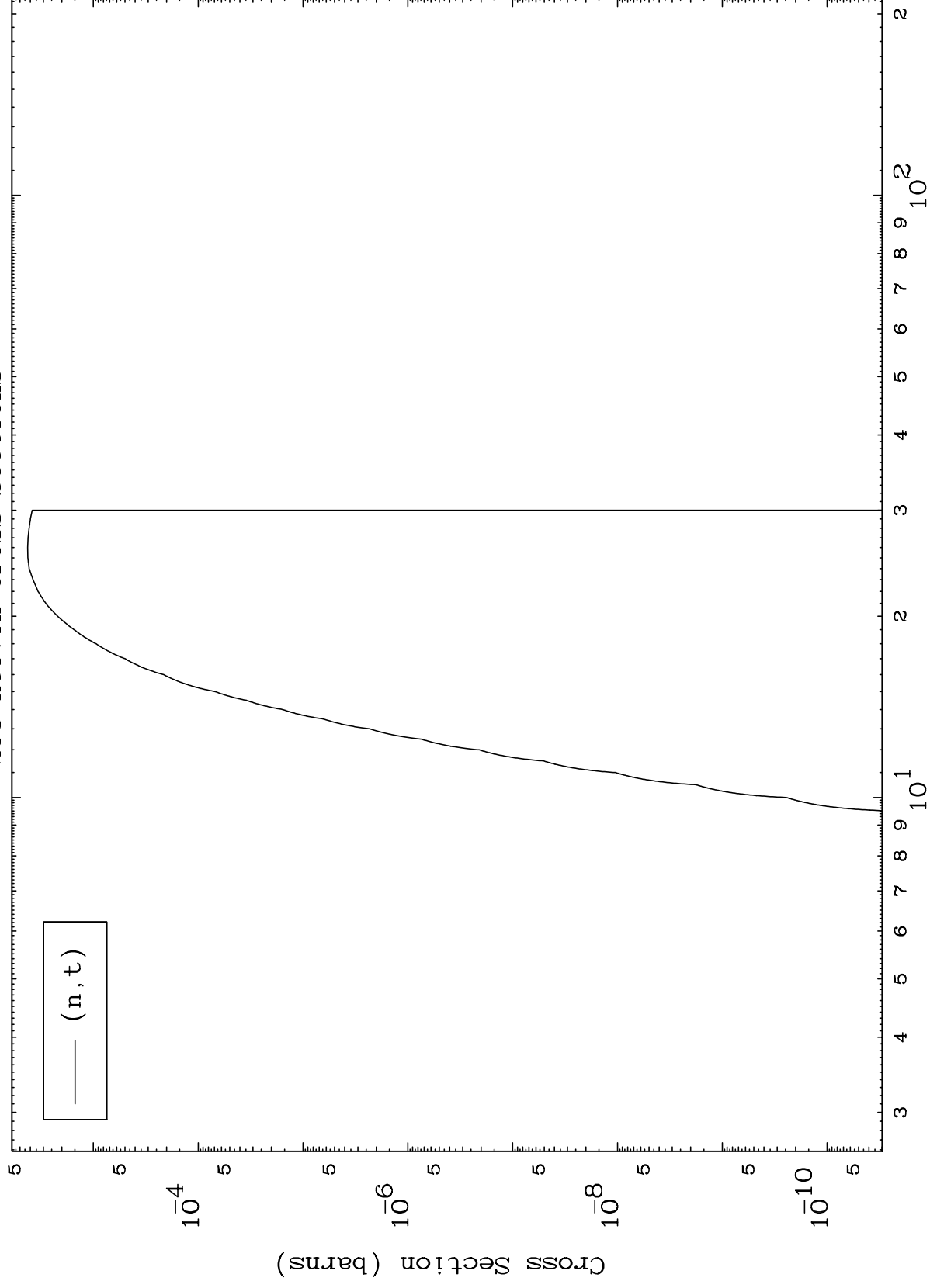
Incident Energy (MeV)

12

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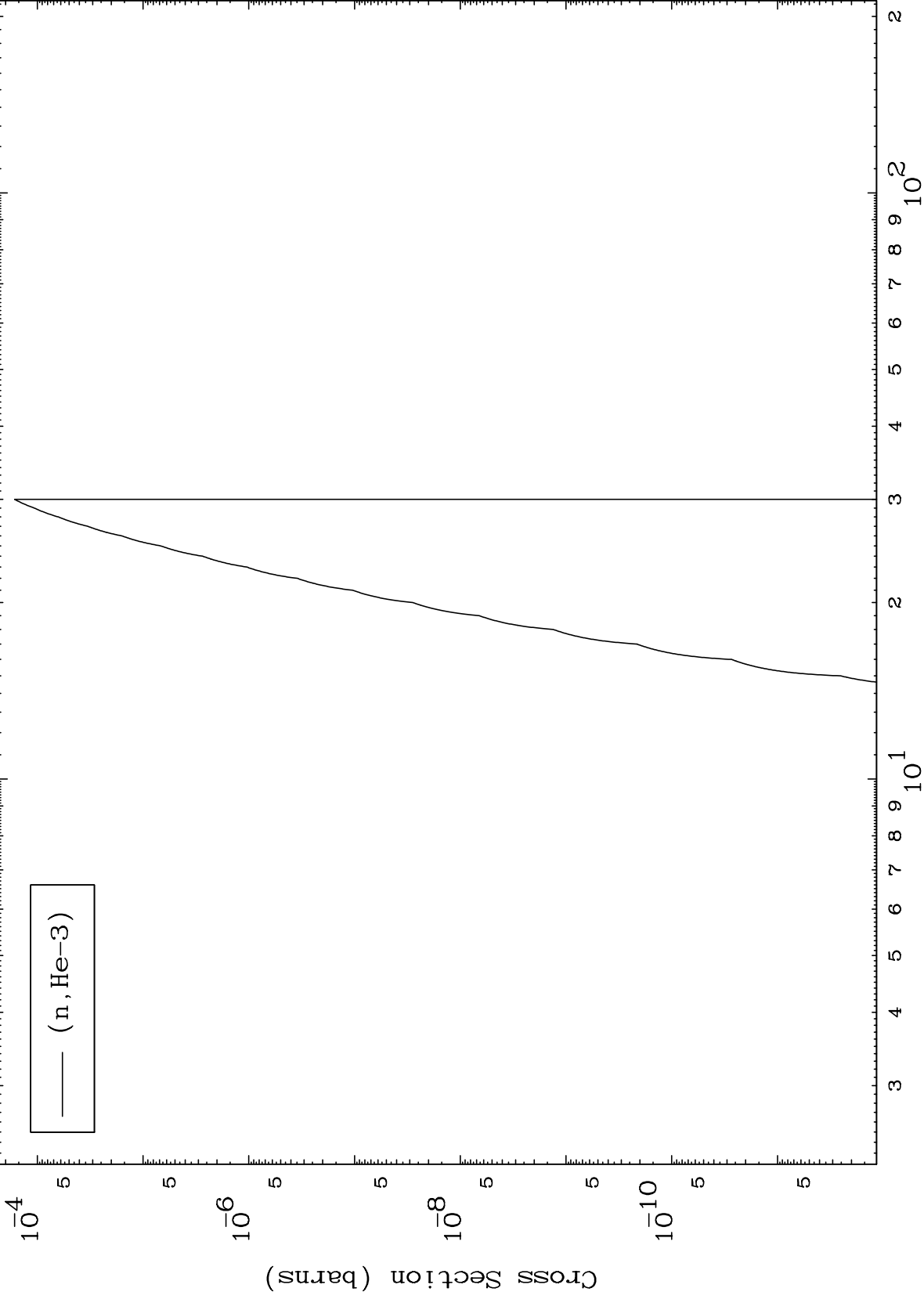
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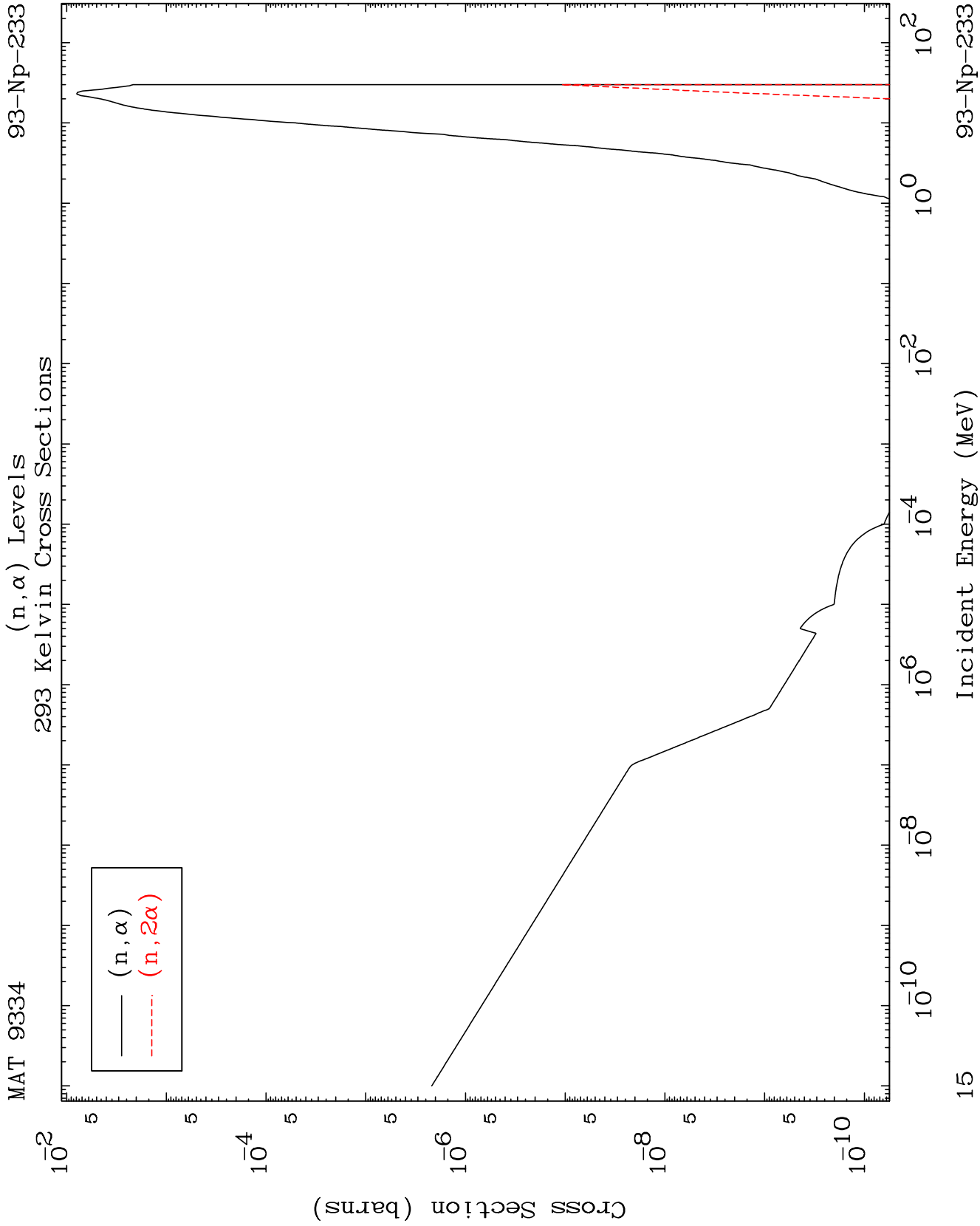
(n,t) Levels
293 Kelvin Cross Sections



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93-Np-233

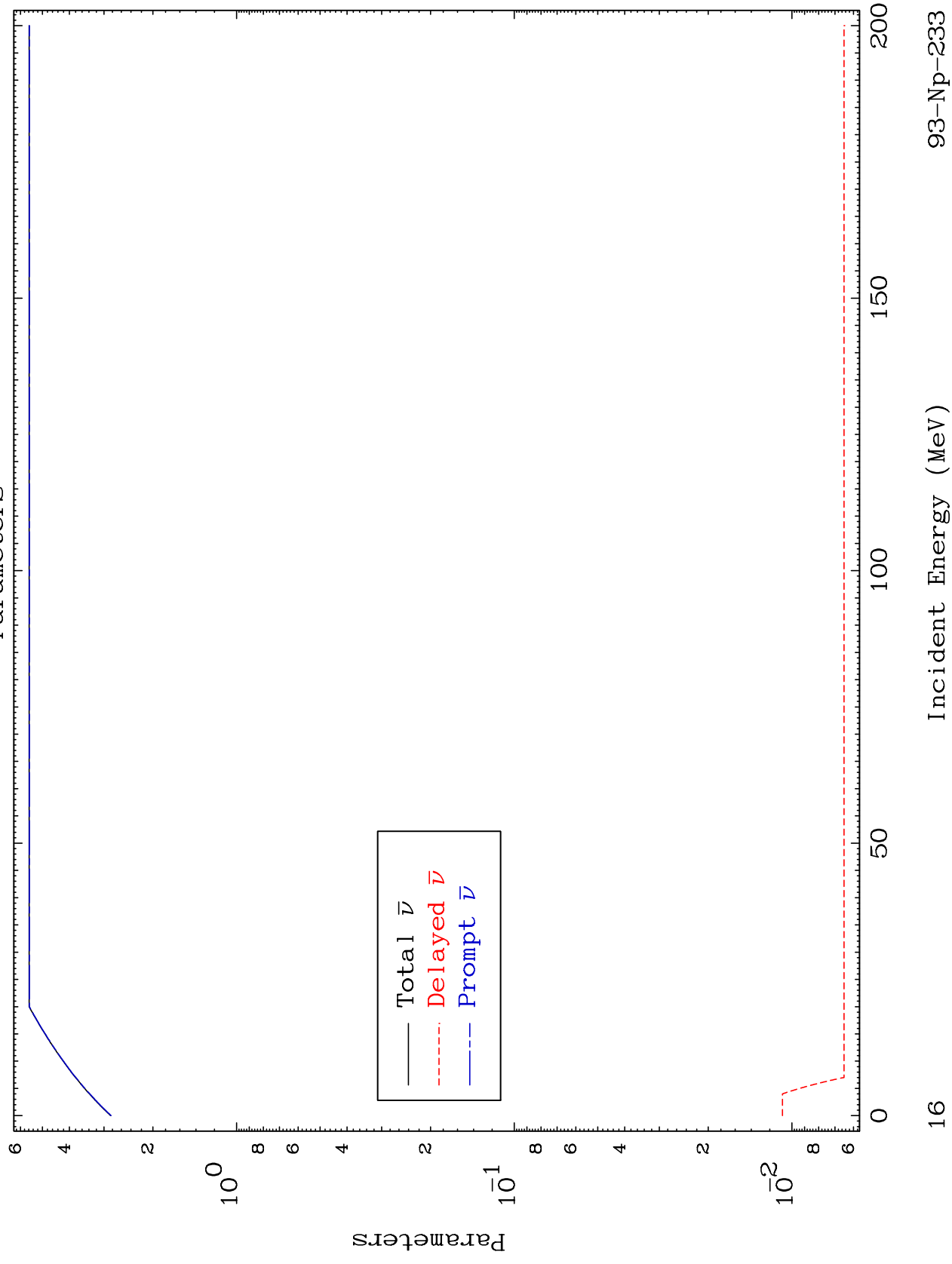


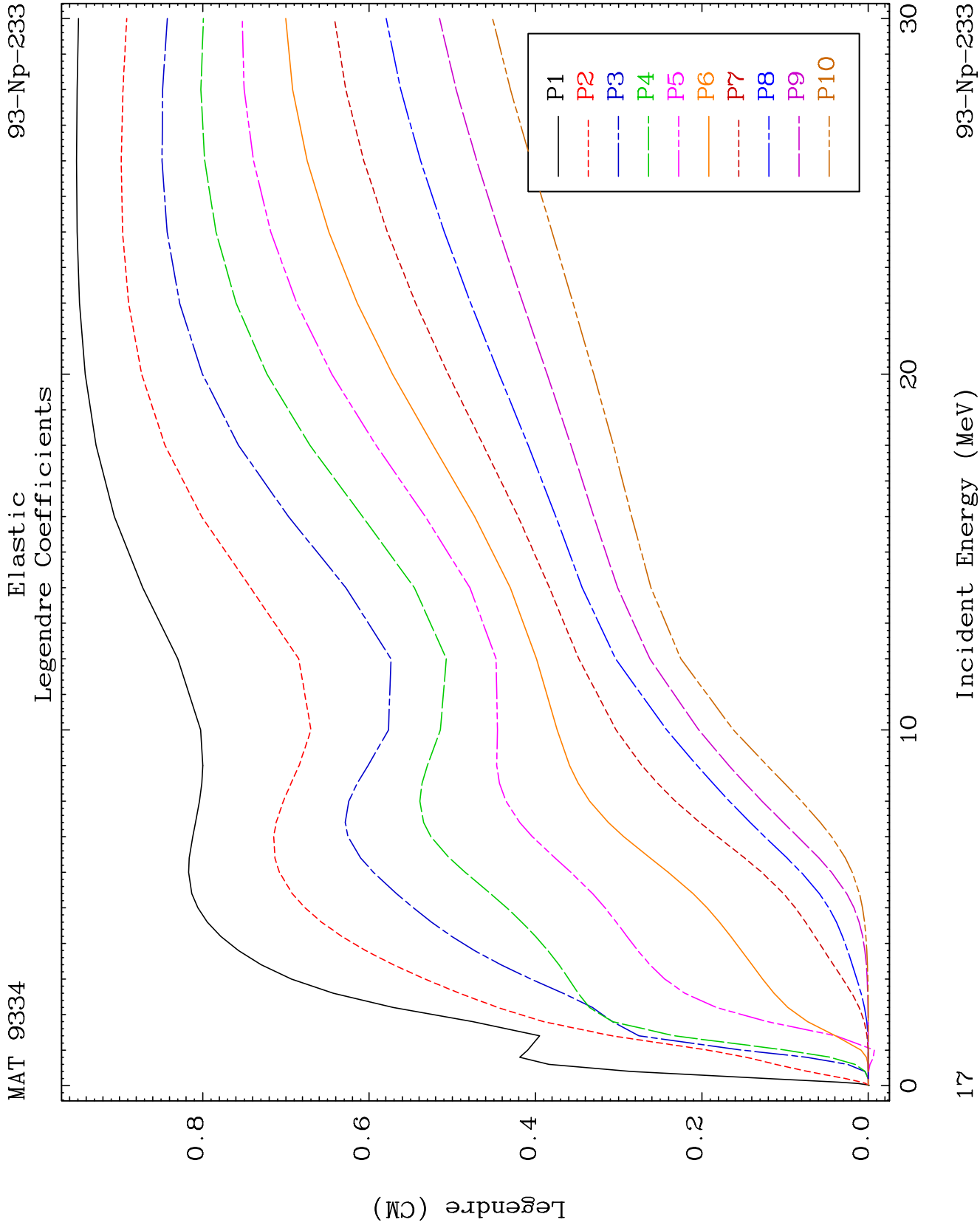


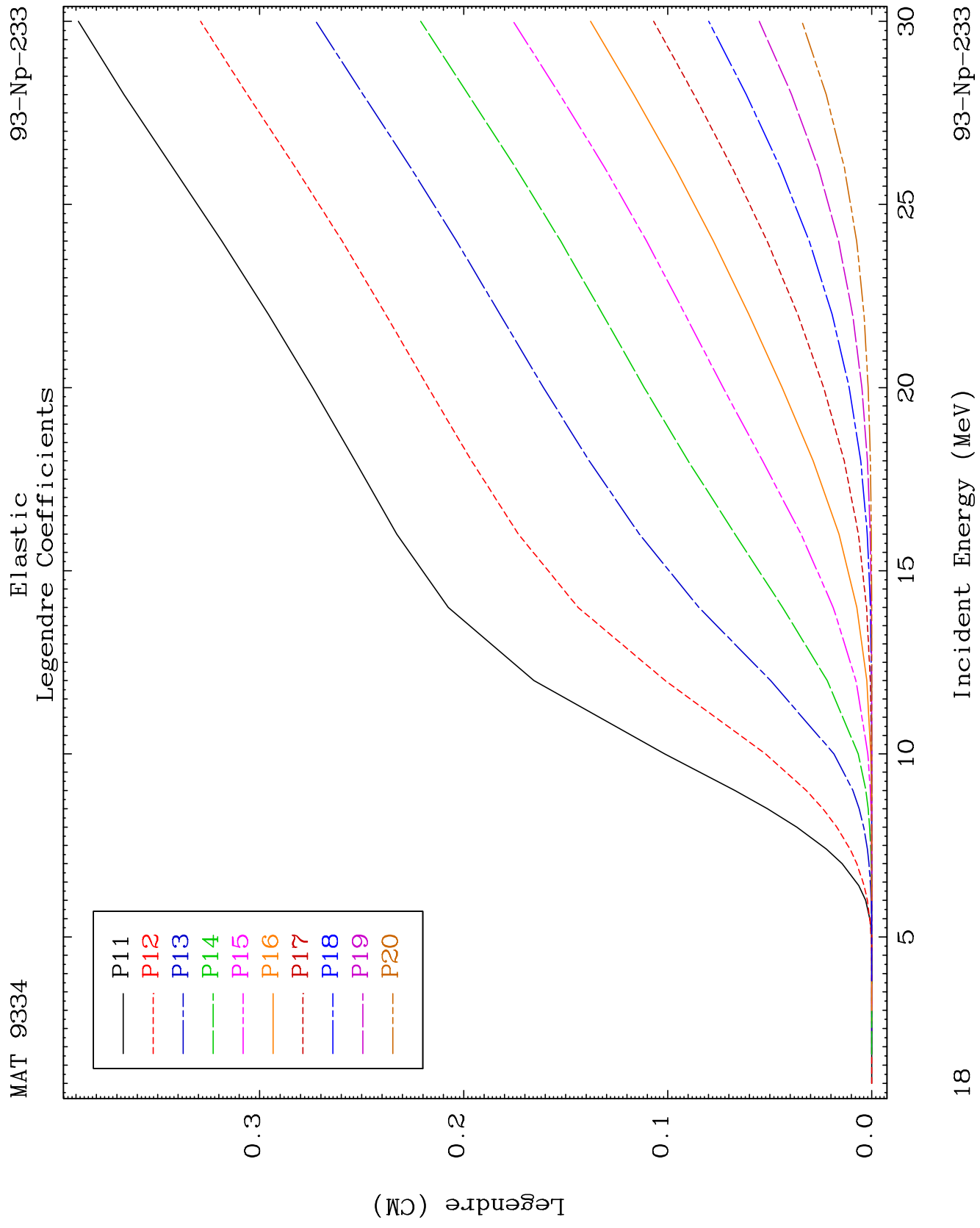
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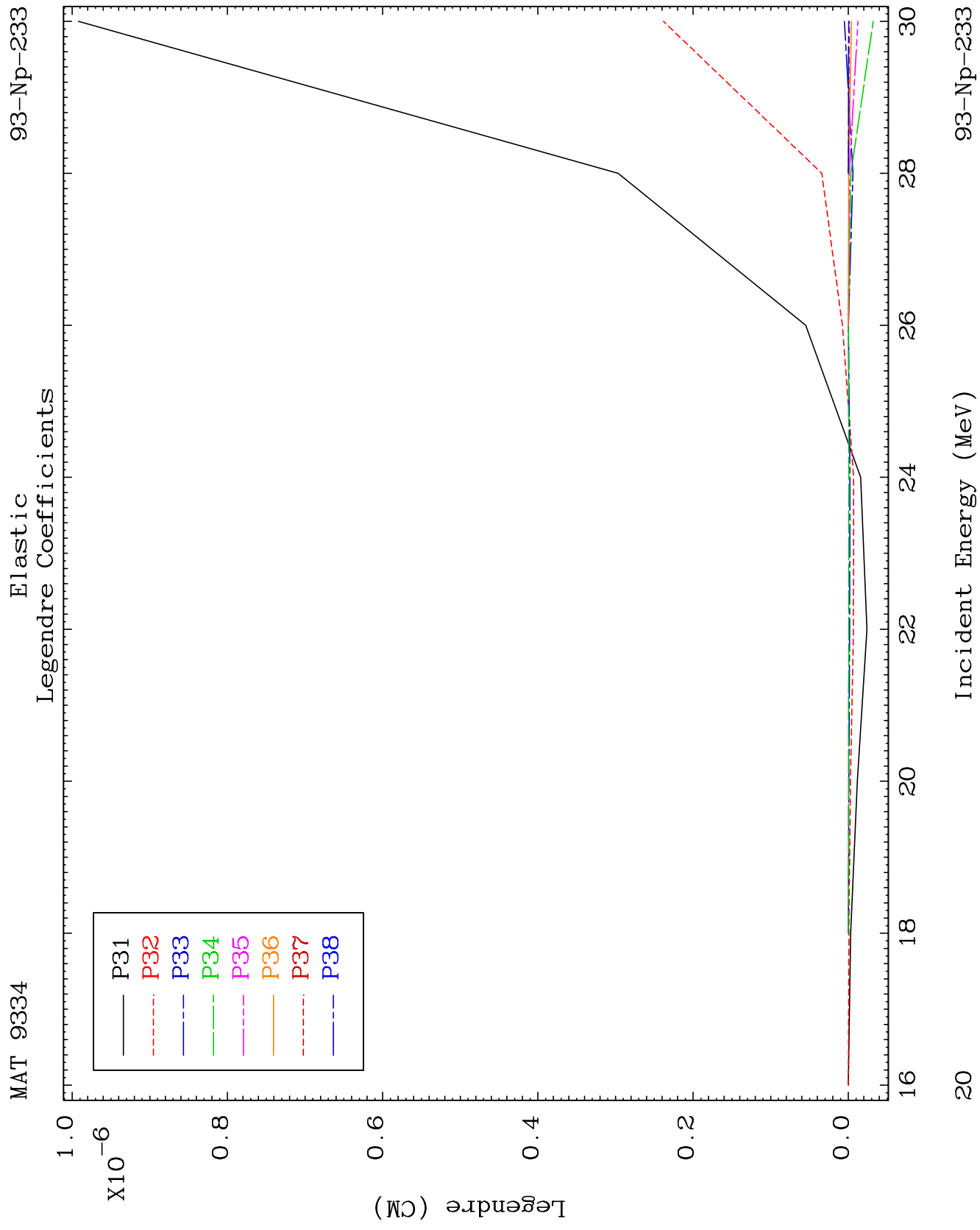
Energy Release Parameters

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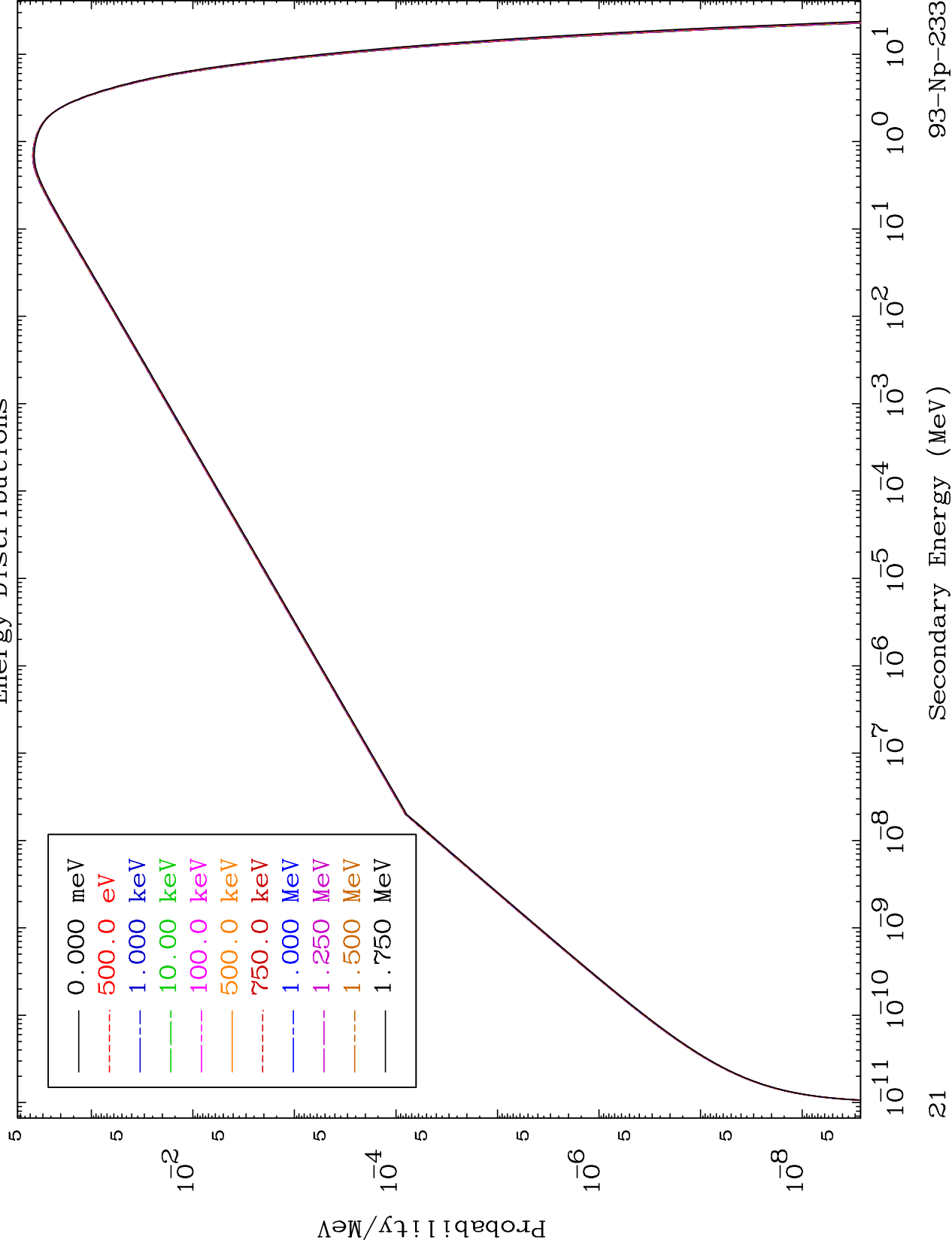


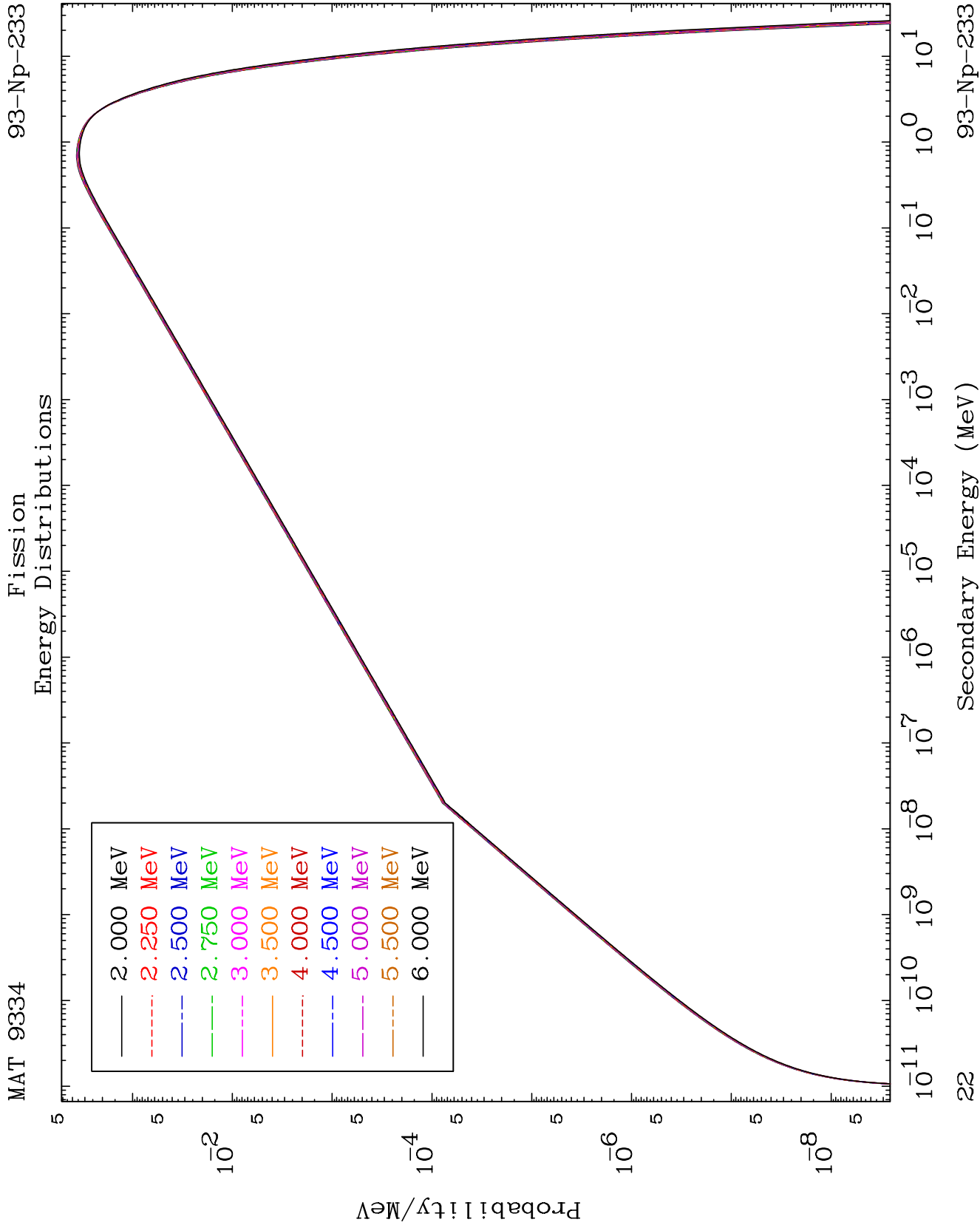


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Fission
Energy Distributions

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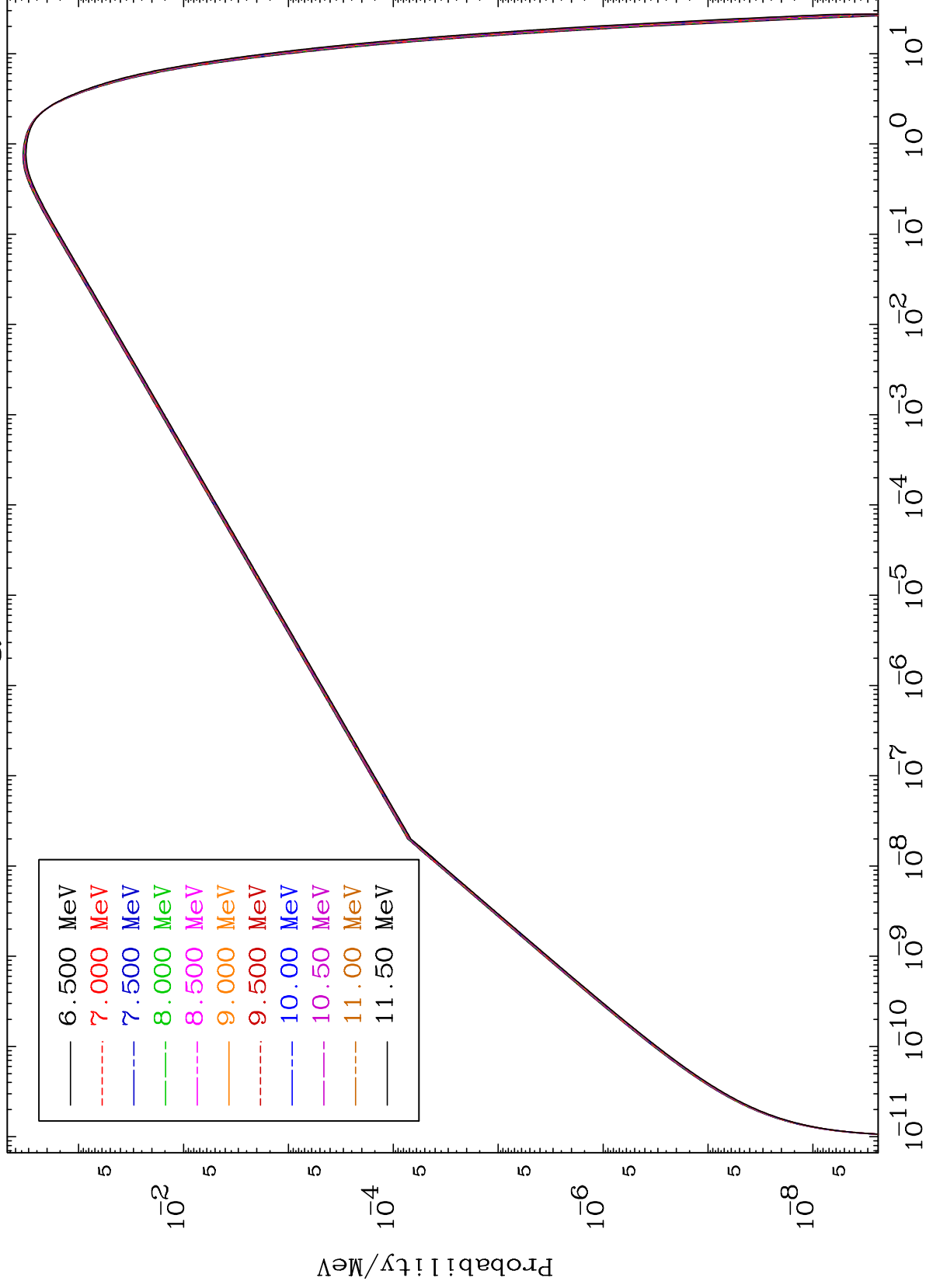




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Fission
Energy Distributions

93-Np-233



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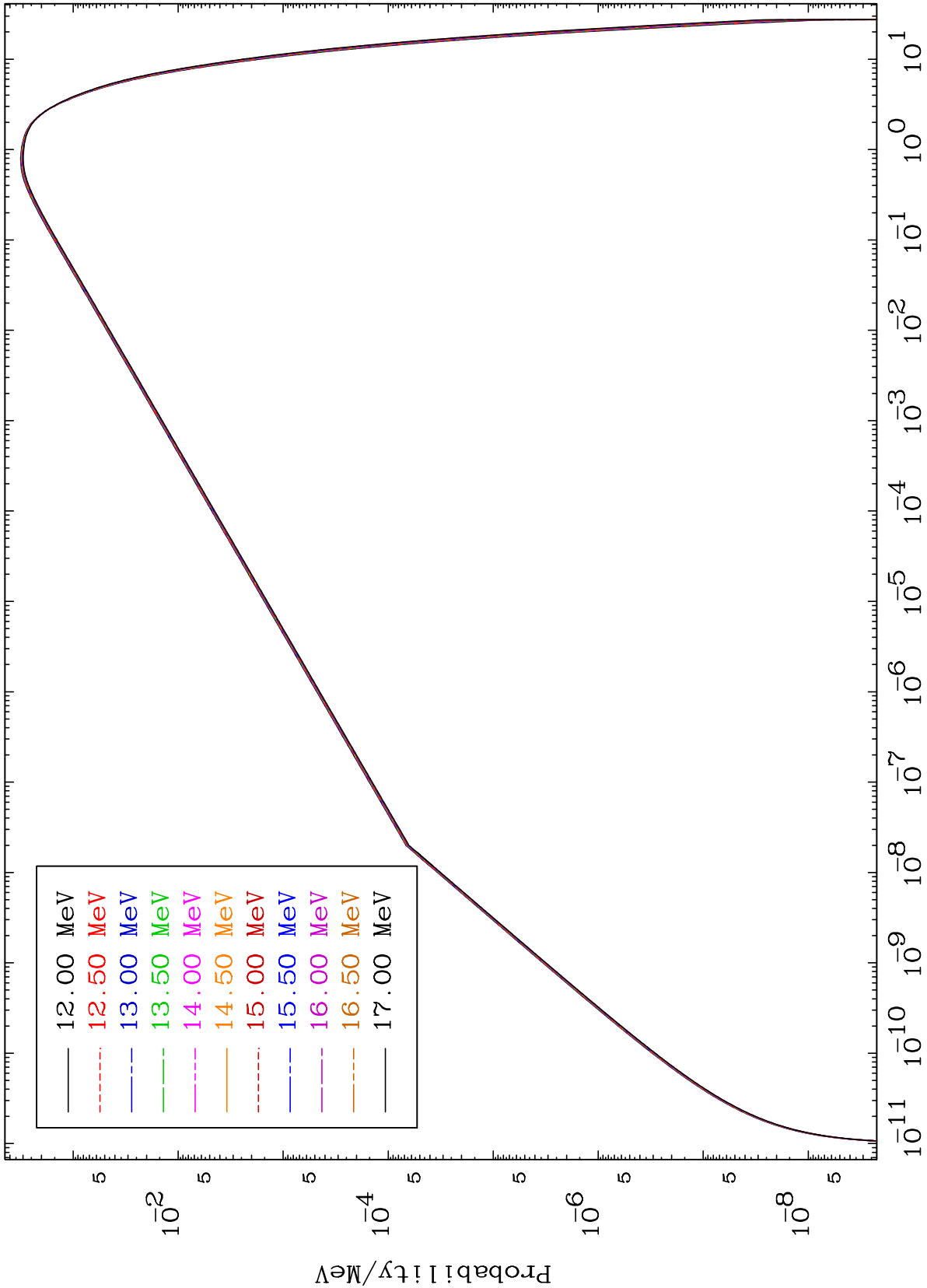
Secondary Energy (MeV)

93-Np-233

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Fission
Energy Distributions

93-Np-233



Fission
Energy Distributions

93-Np-233

