

Program EVALPLOT  
(Version 2021-1)

by

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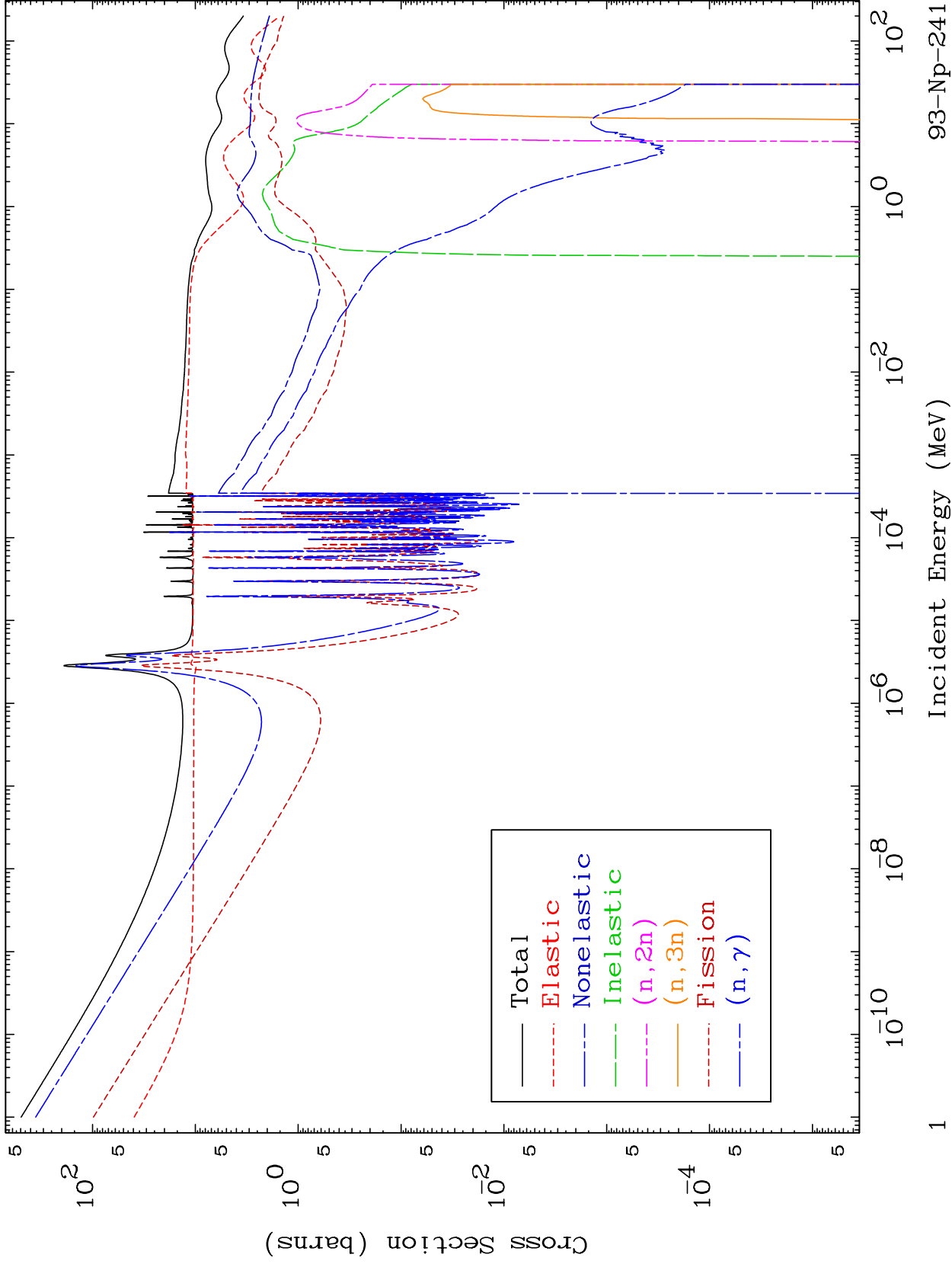
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Press Mouse Button to Start

MAT 9358

Neutron Major  
293 Kelvin Cross Sections

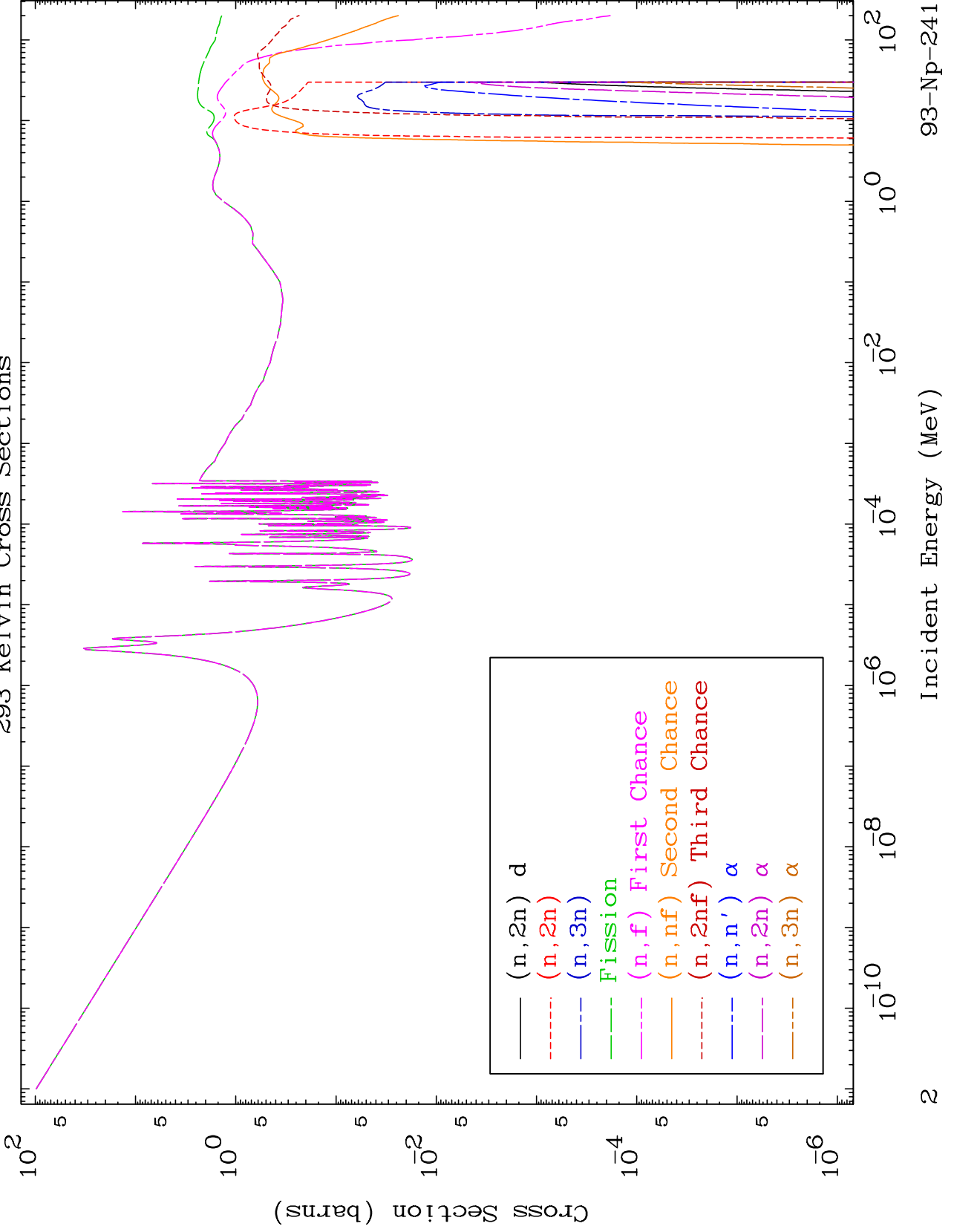
93-Np-241



MAT 9358

Neutron Absorption  
293 Kelvin Cross Sections

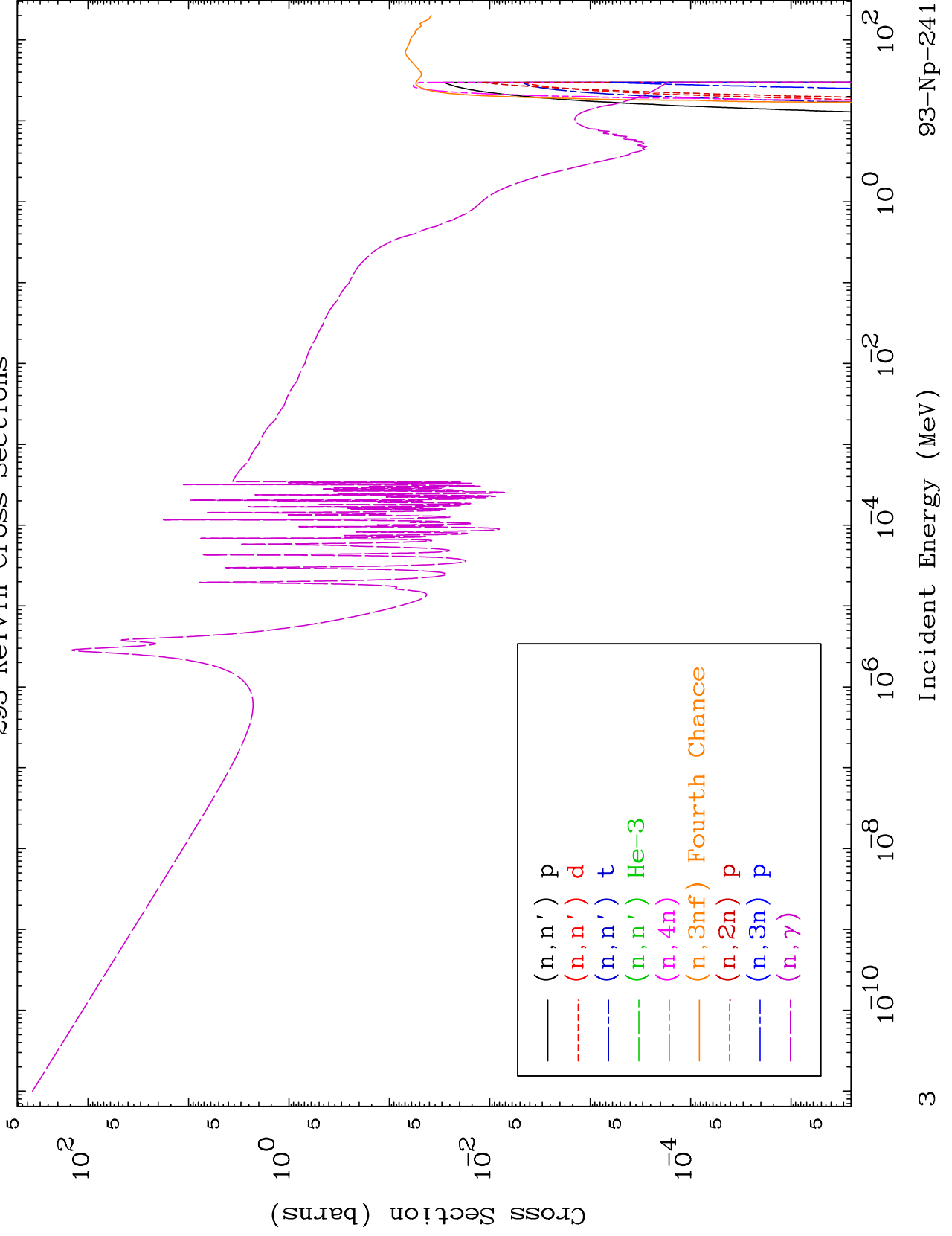
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MAT 9358

Neutron Absorption  
293 Kelvin Cross Sections

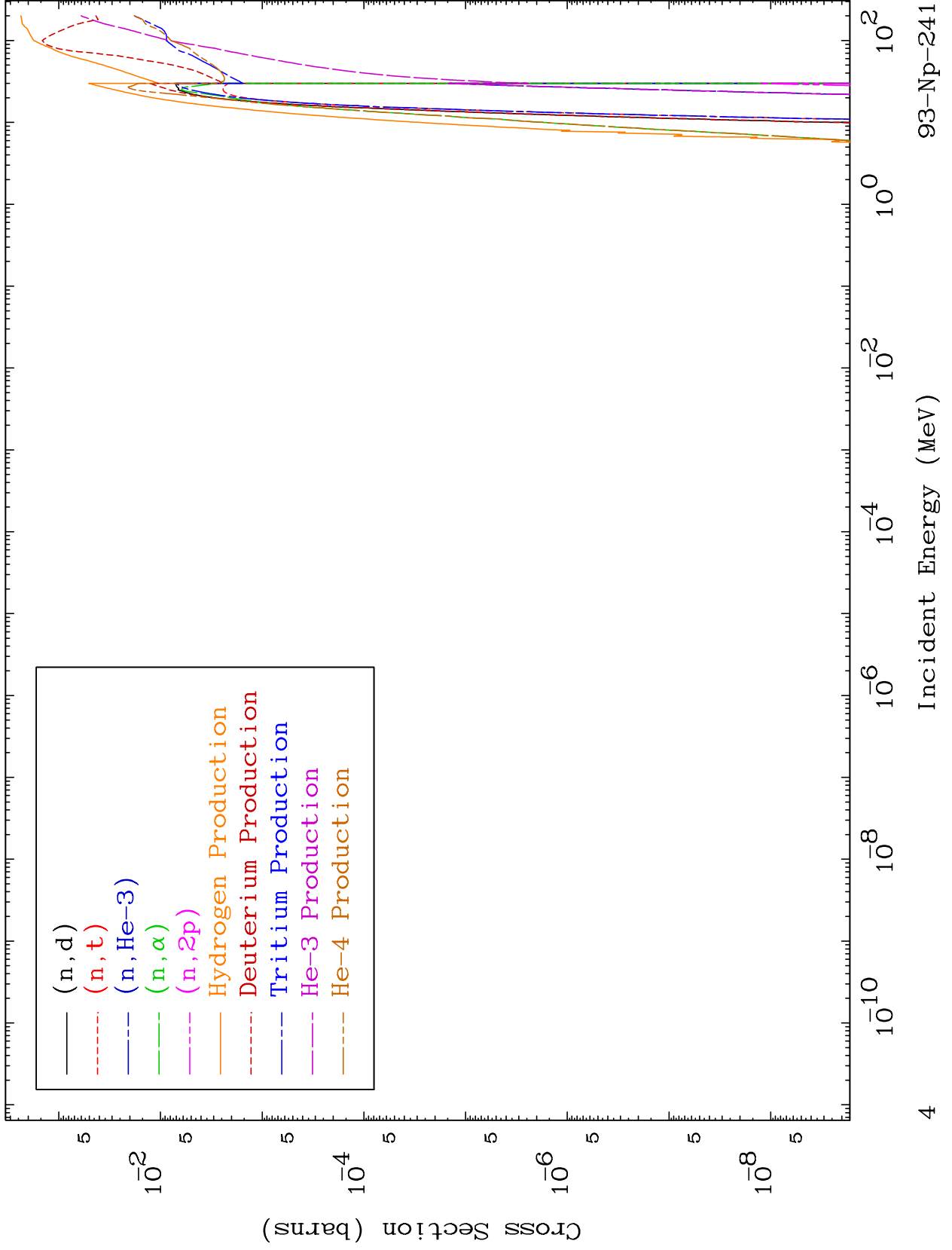
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Neutron Absorption  
293 Kelvin Cross Sections

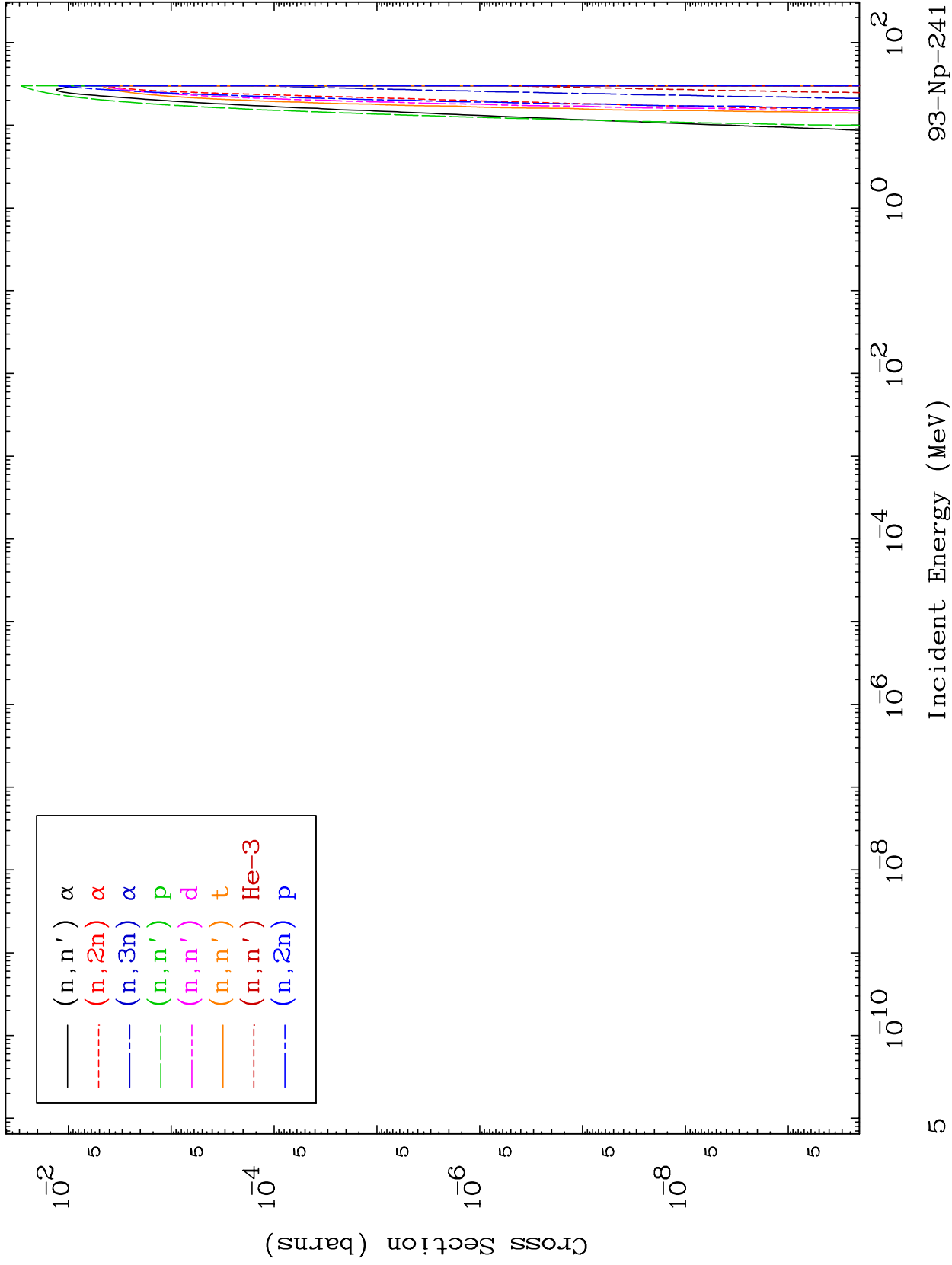
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Charged Particle  
293 Kelvin Cross Sections

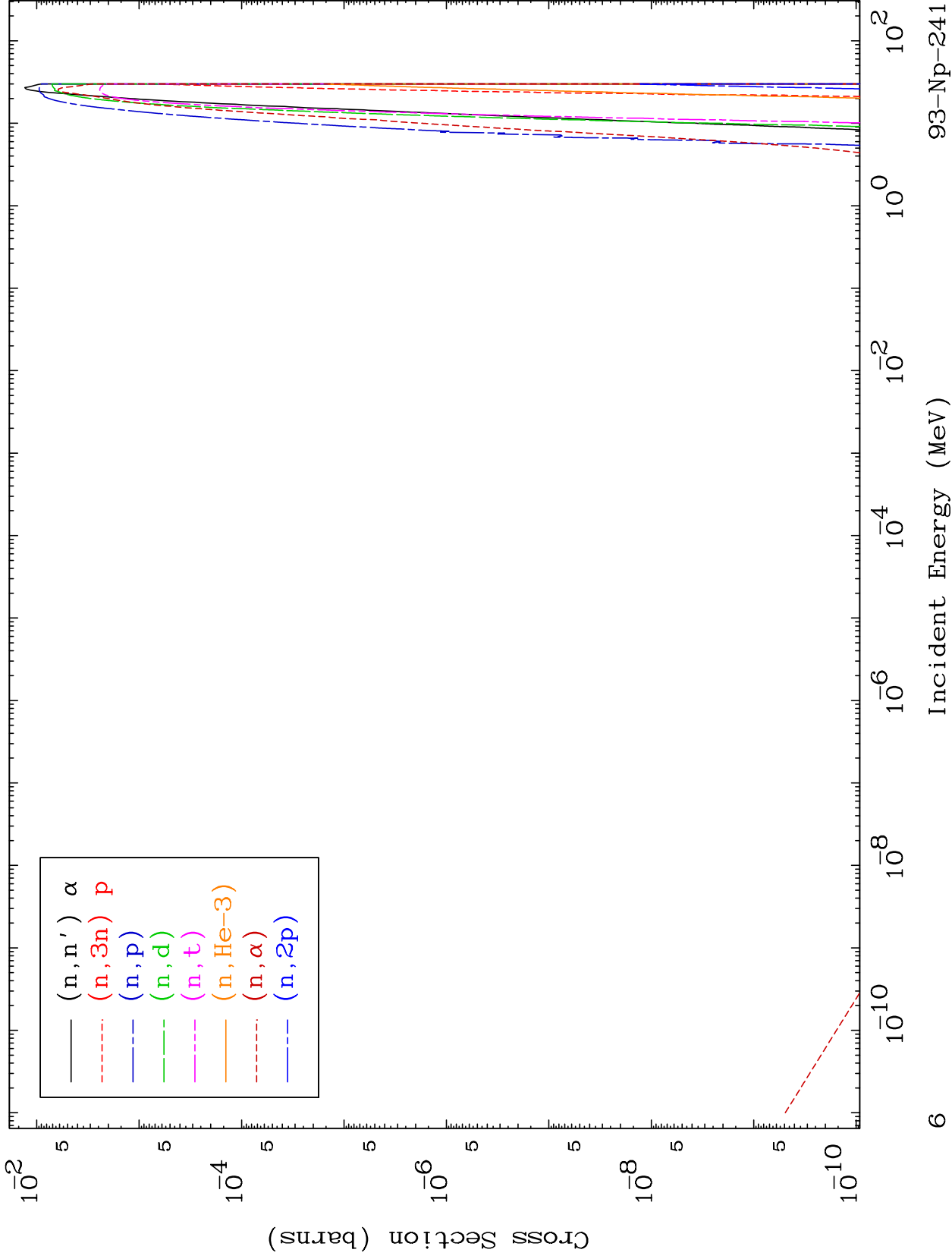
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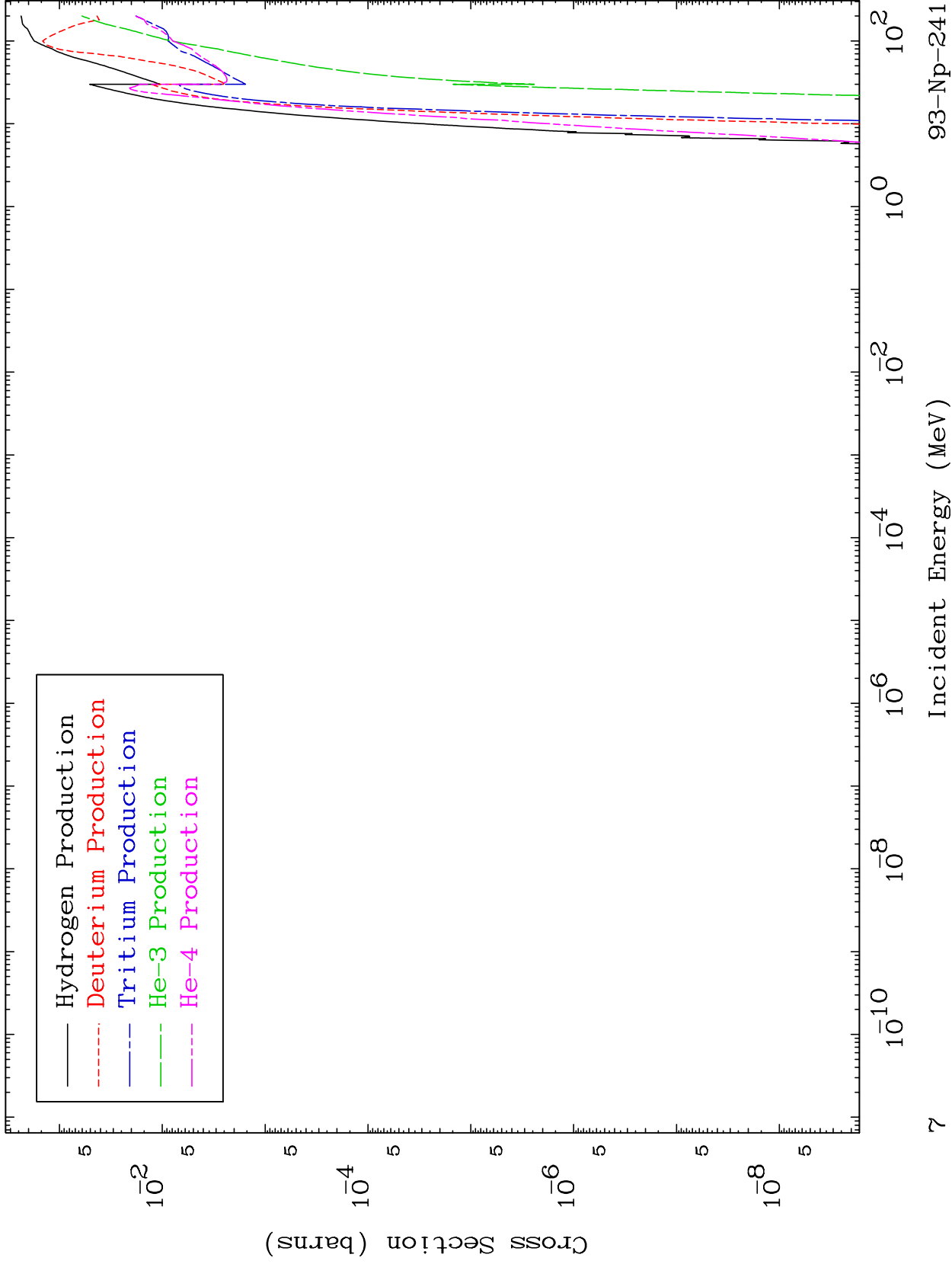


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Charged Particle  
293 Kelvin Cross Sections

93-Np-241



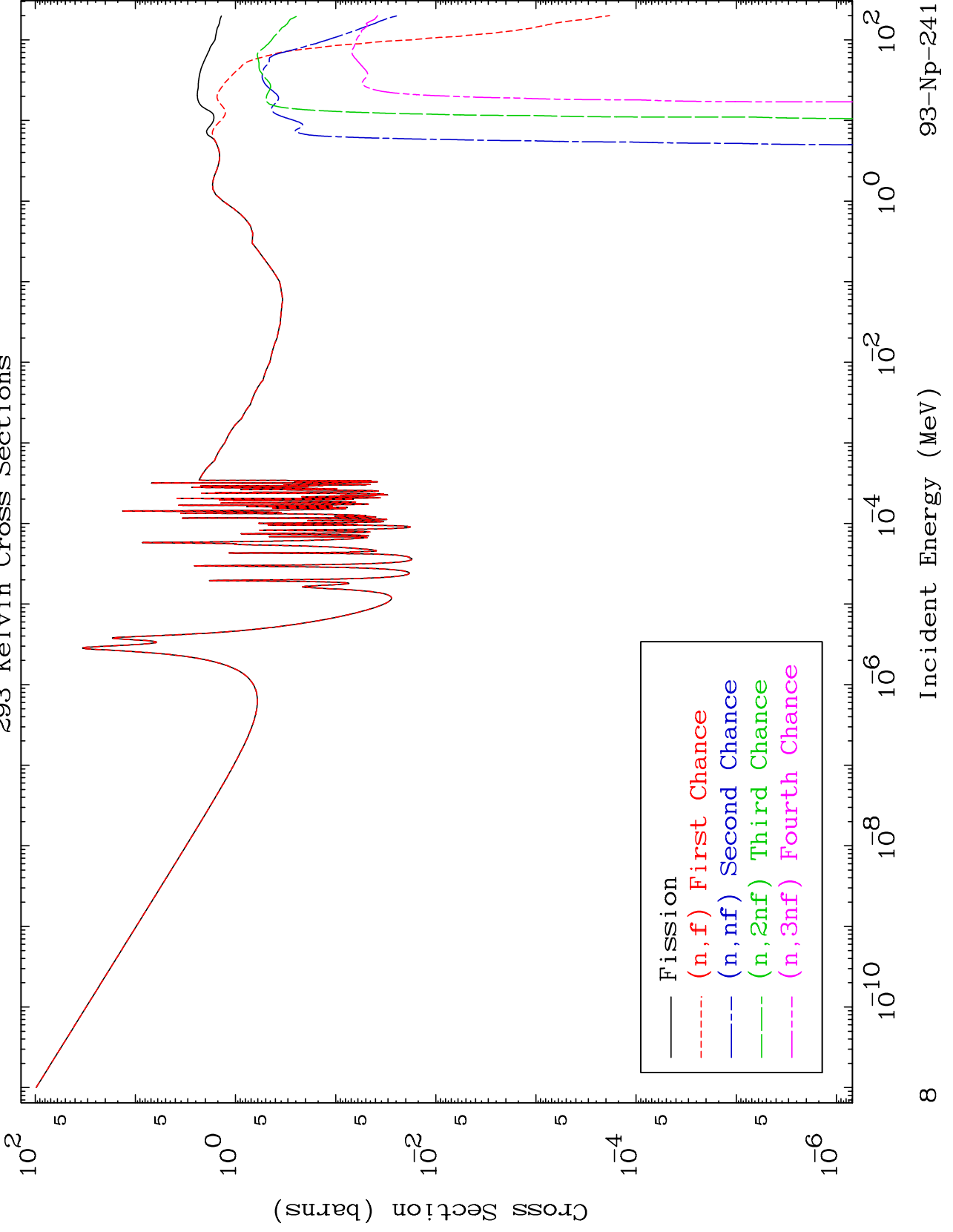




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Fission  
293 Kelvin Cross Sections

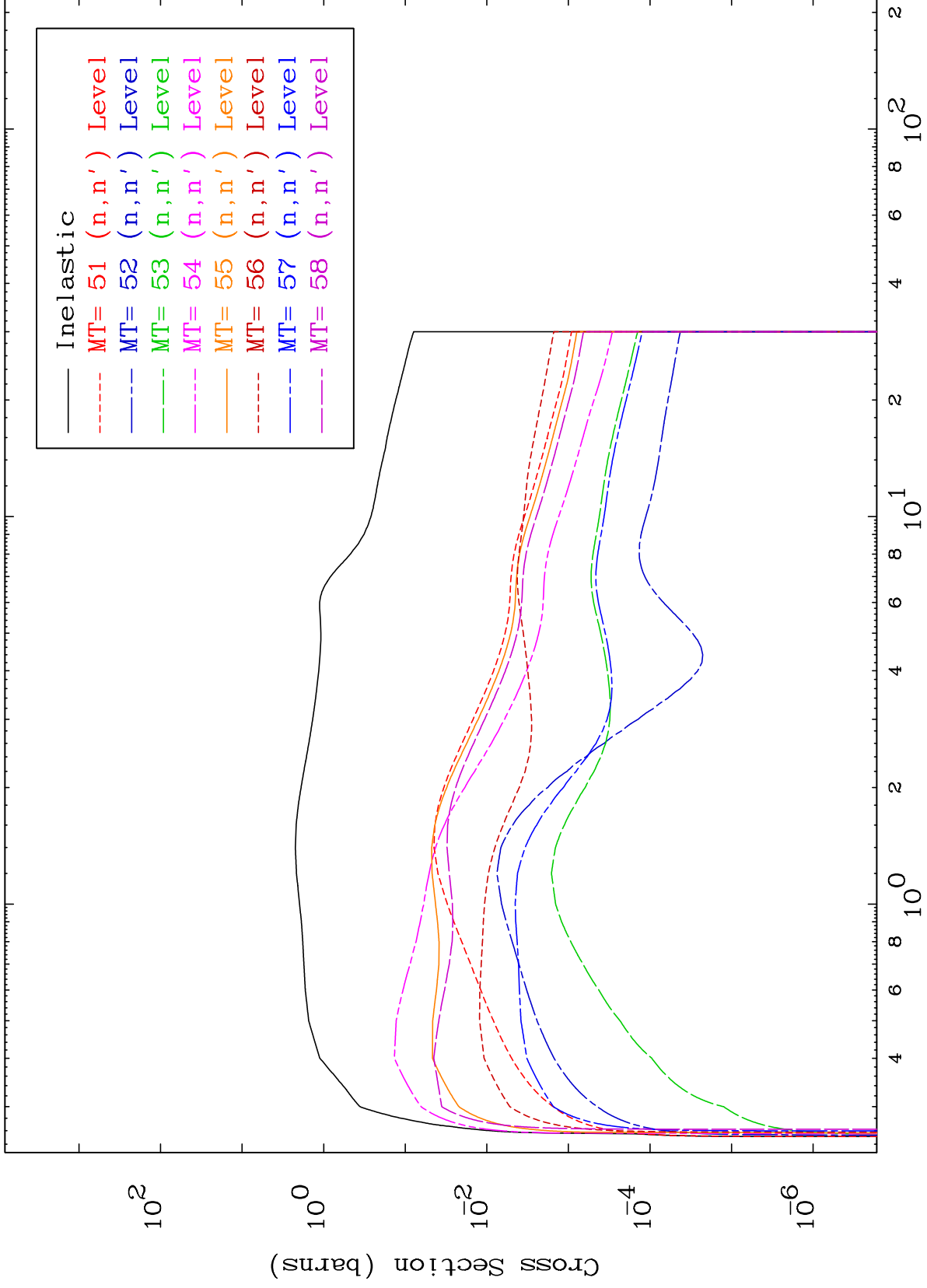
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(n,n') Levels  
293 Kelvin Cross Sections

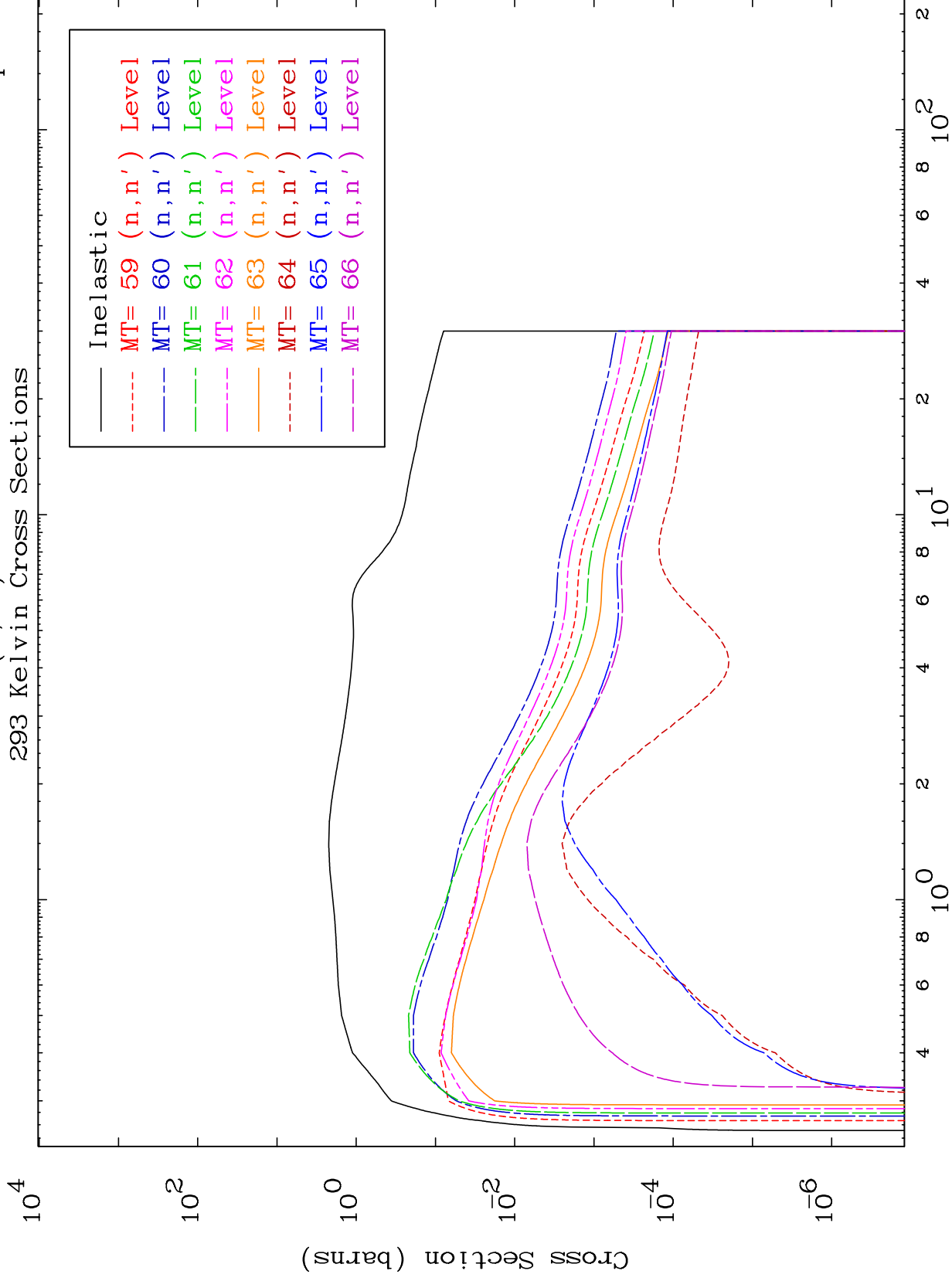
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(n,n') Levels  
293 Kelvin Cross Sections

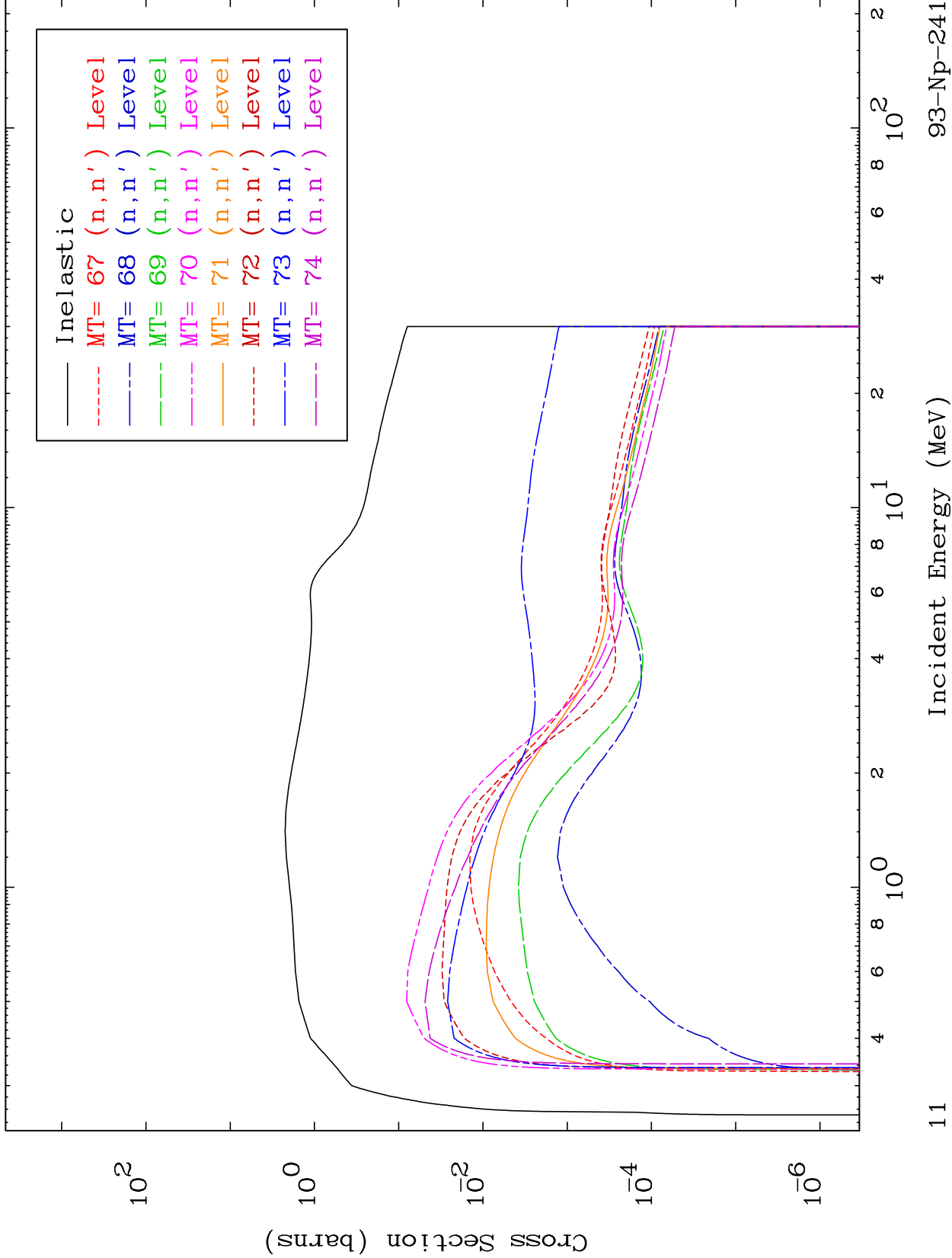
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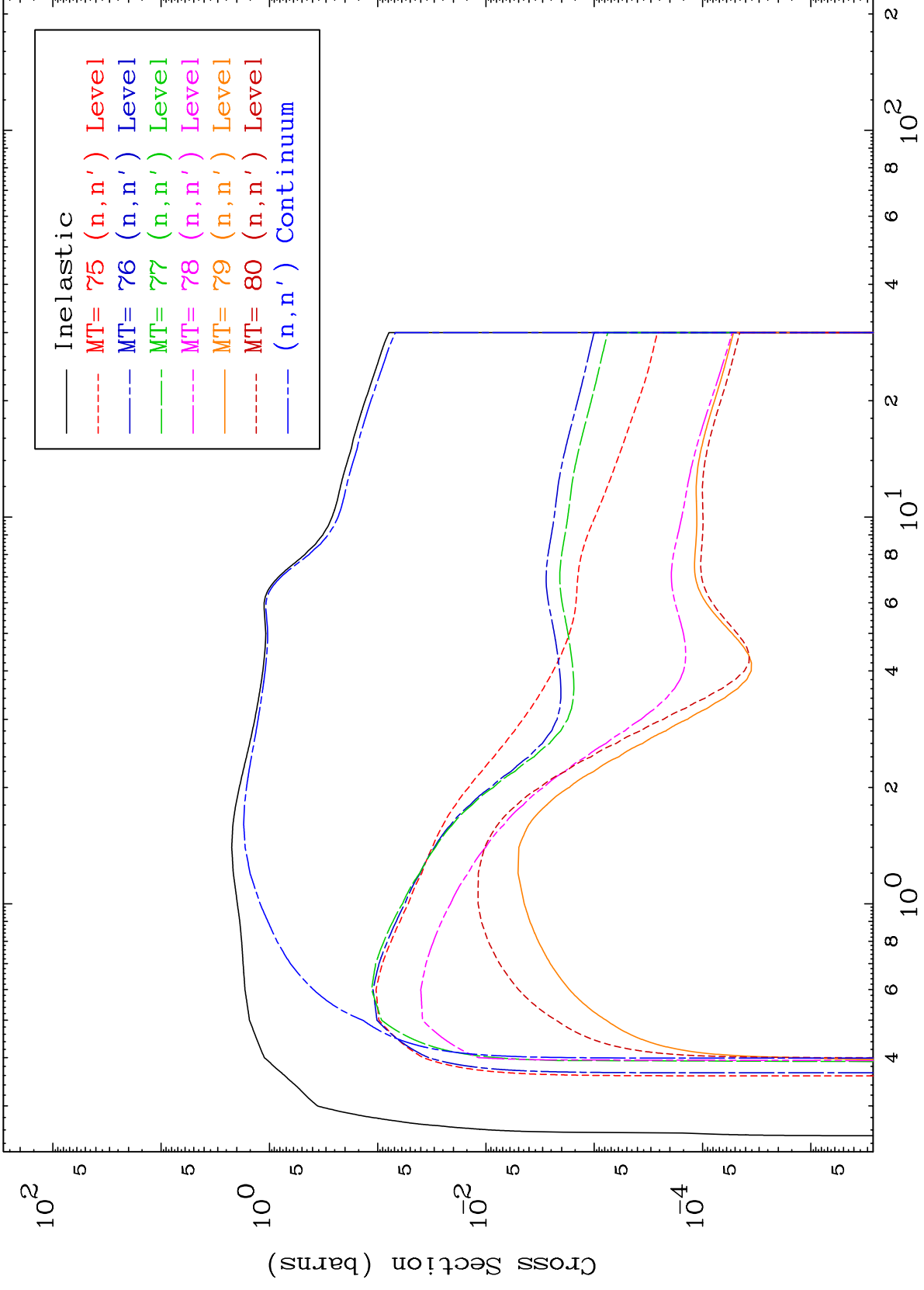


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Incident Energy (MeV)

93-Np-241

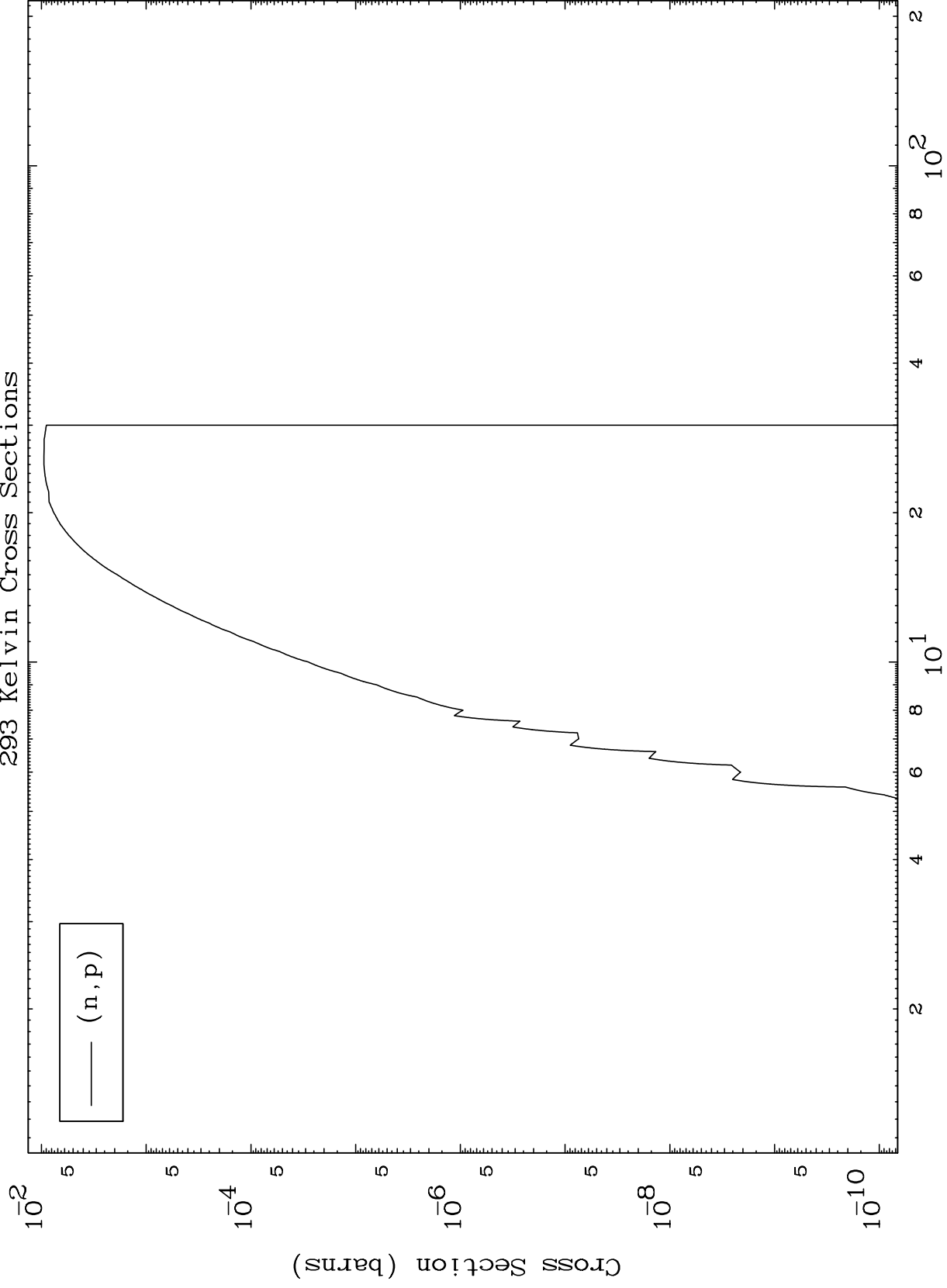




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(n,p) Levels  
293 Kelvin Cross Sections

93-Np-241



13

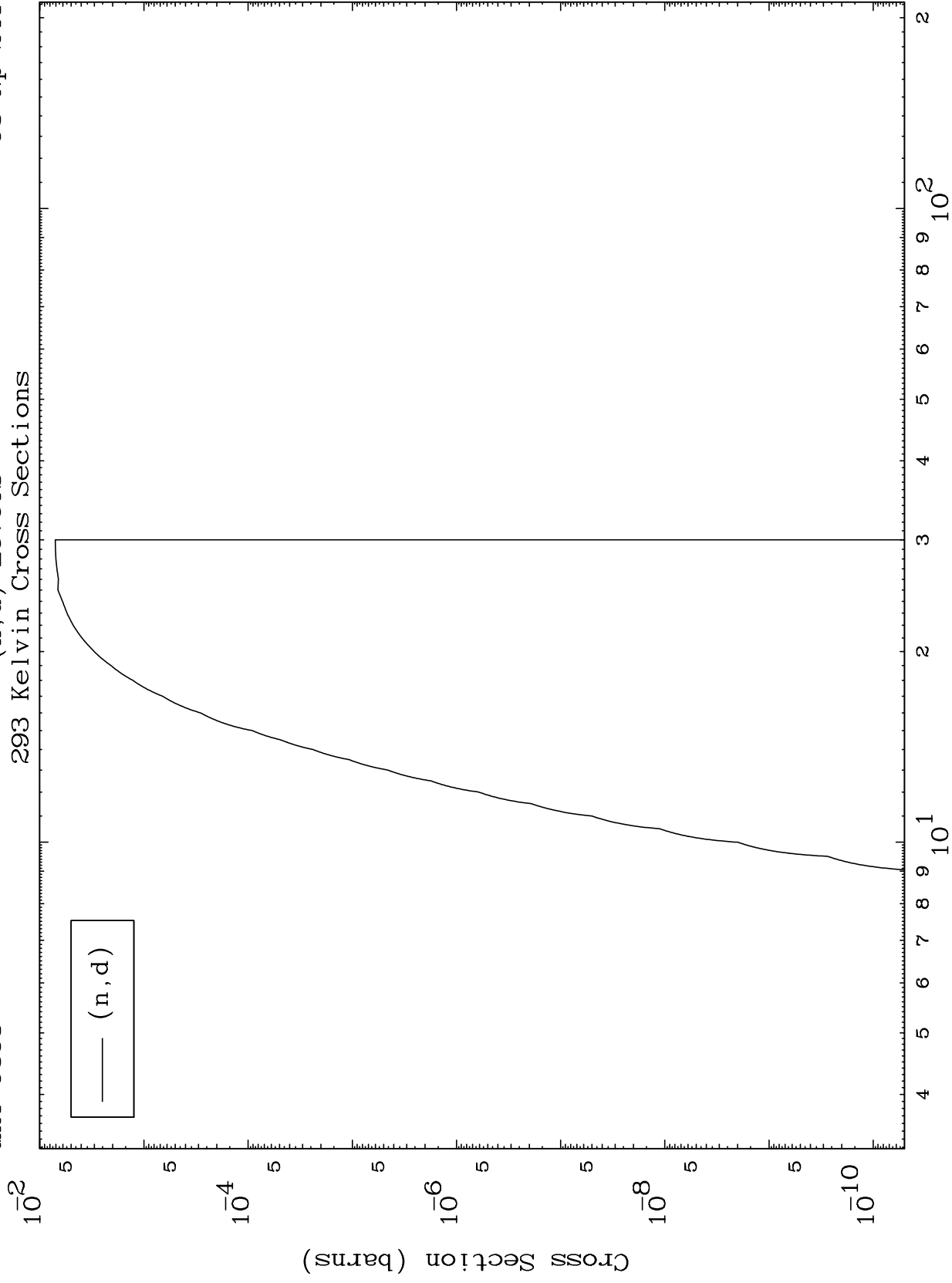
Incident Energy (MeV)

93-Np-241

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(n,d) Levels  
293 Kelvin Cross Sections

93-Np-241



14

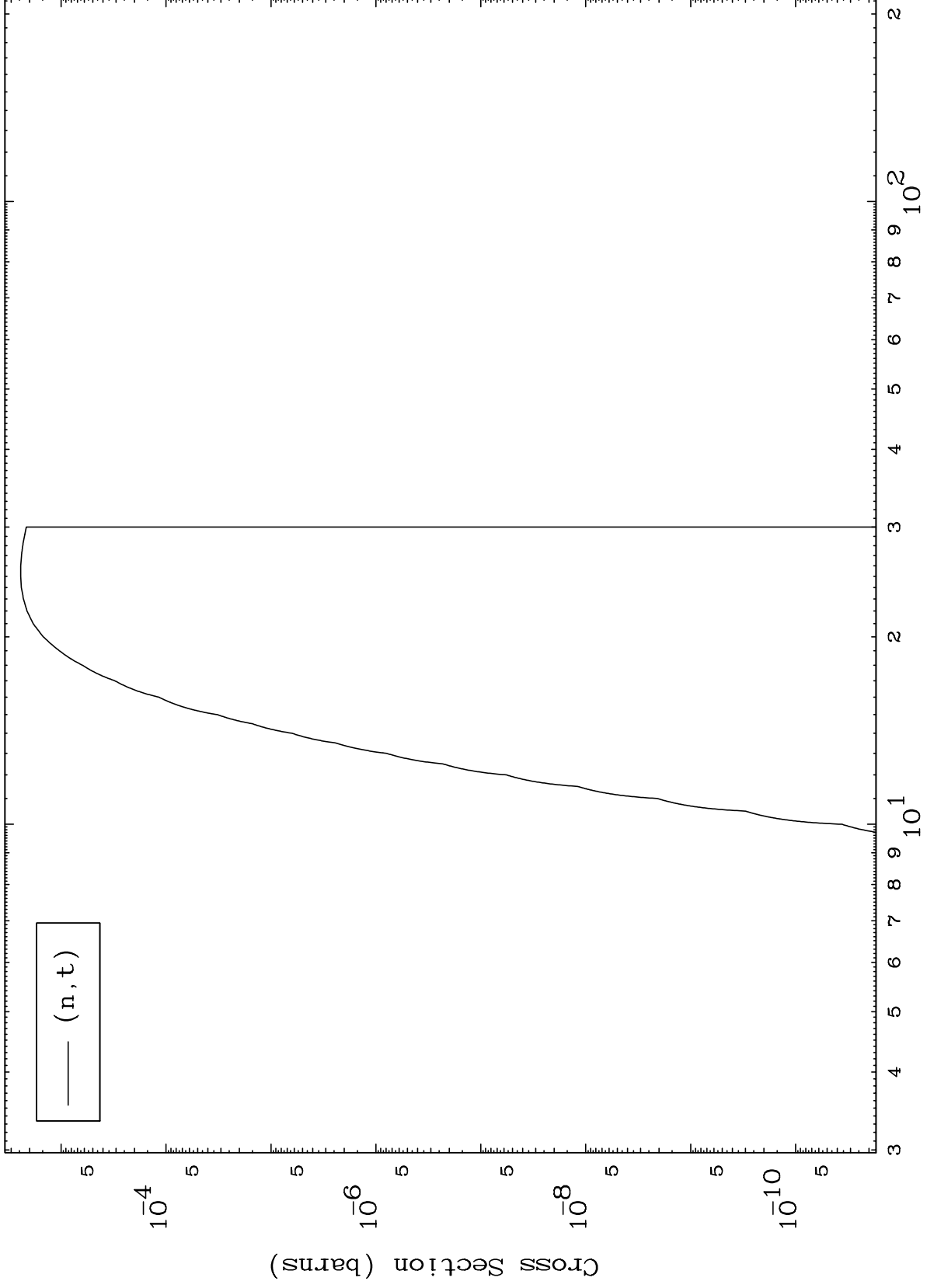
Incident Energy (MeV)

93-Np-241

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(n,t) Levels  
293 Kelvin Cross Sections

93-Np-241

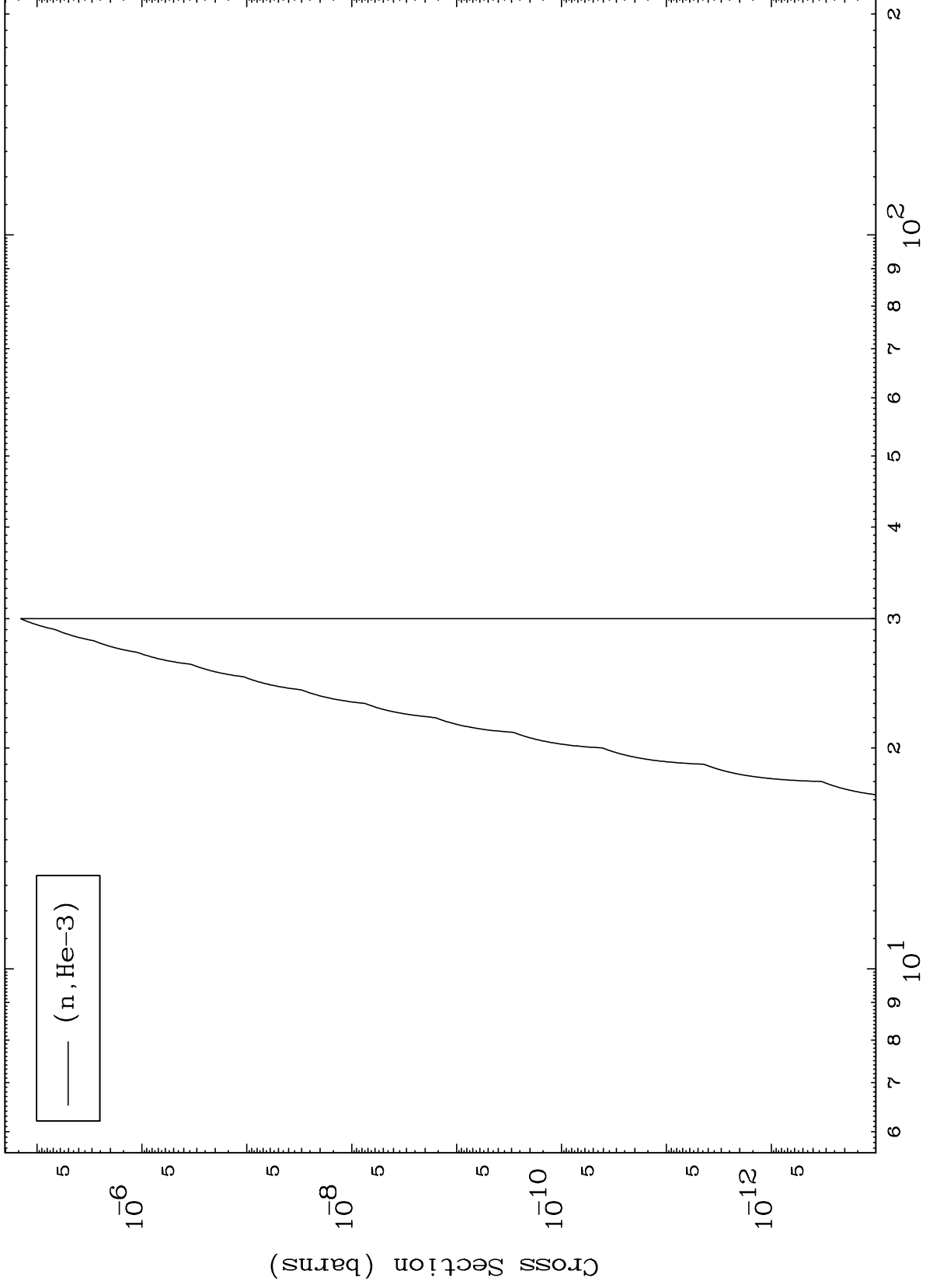




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(n,He3) Levels  
293 Kelvin Cross Sections

93-Np-241



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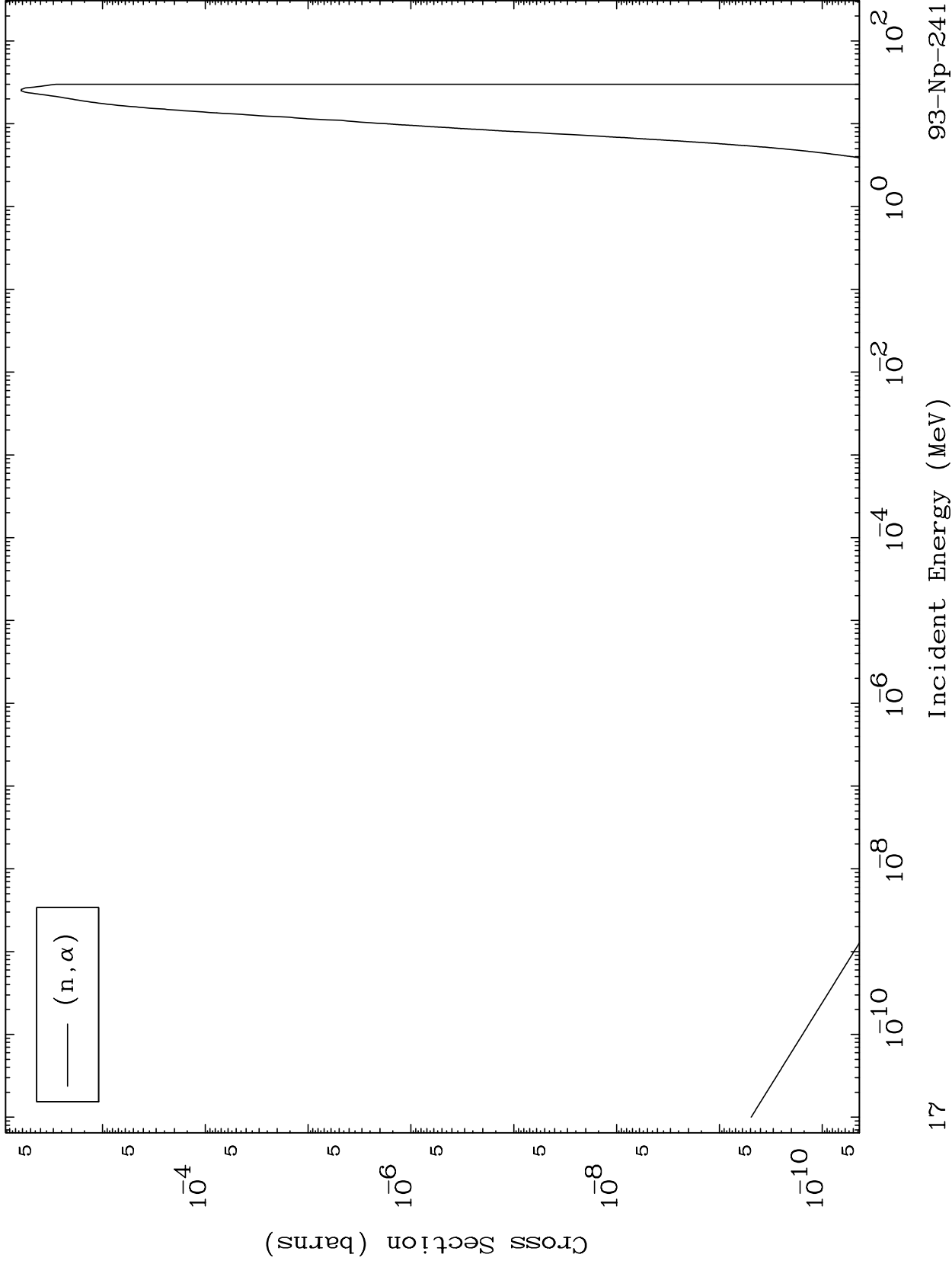
Incident Energy (MeV)

93-Np-241

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

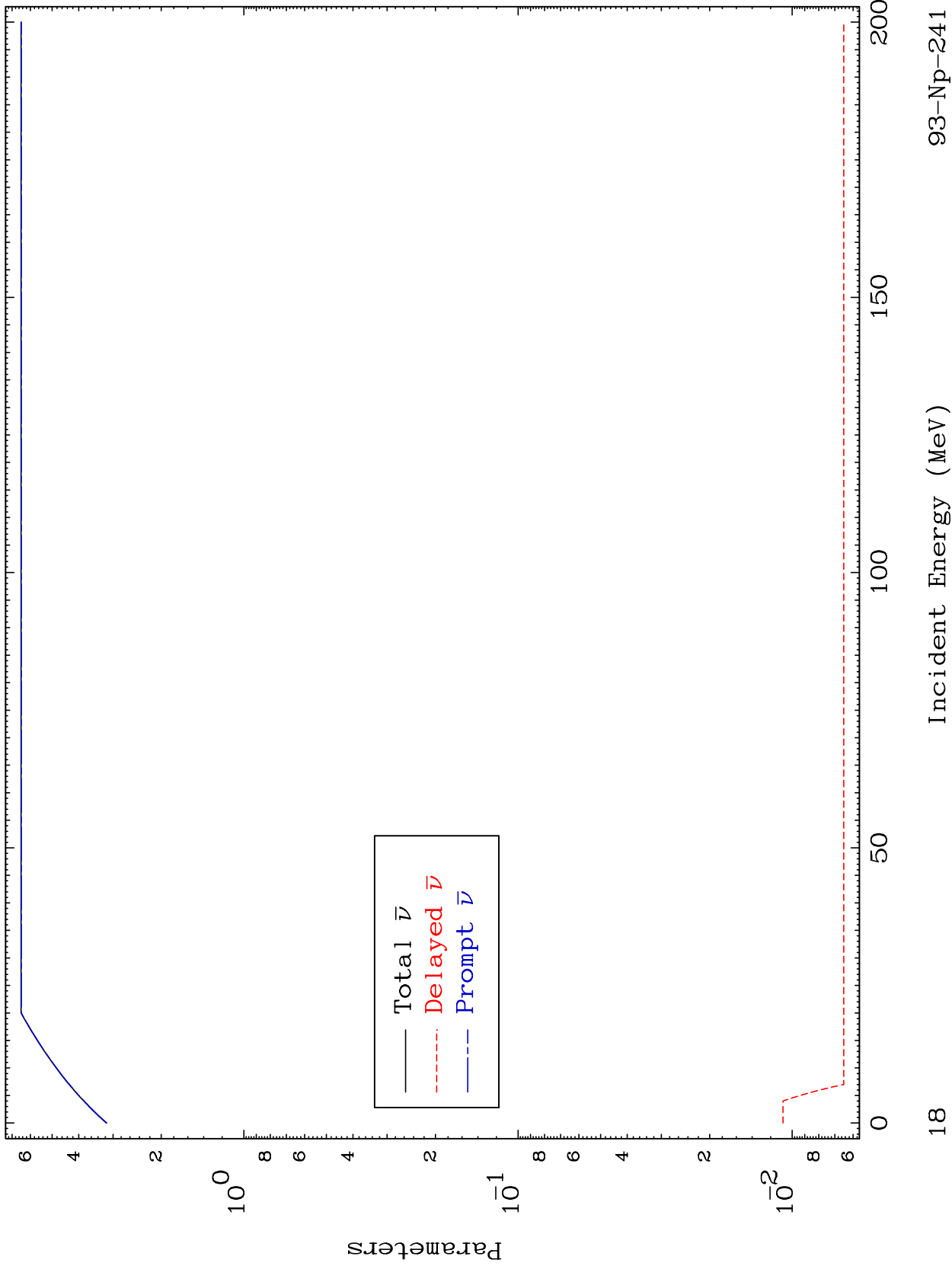
93-Np-241



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Energy Release  
Parameters

93-Np-241



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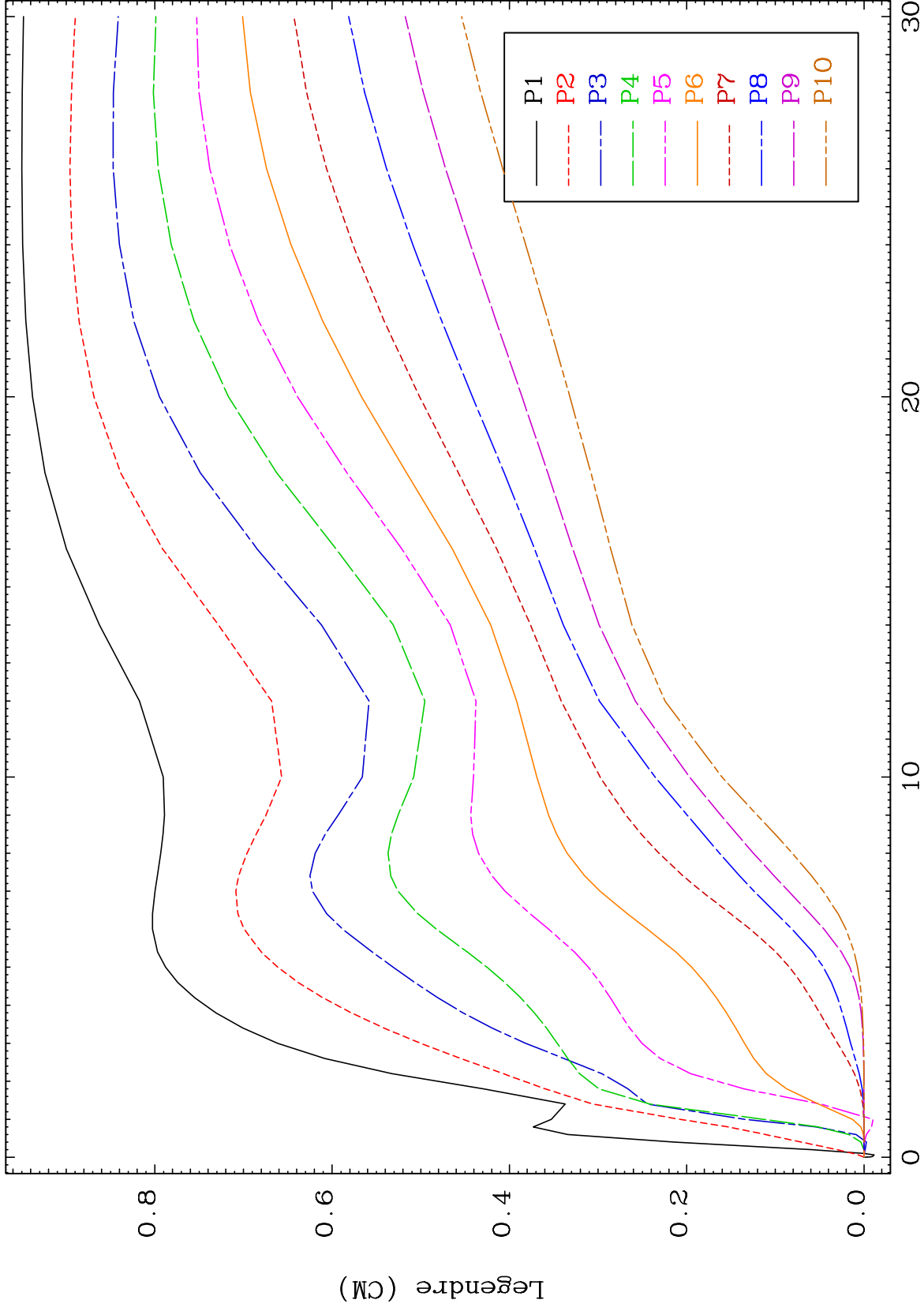
Incident Energy (MeV)

93-Np-241

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Elastic Legendre Coefficients

93-Np-241



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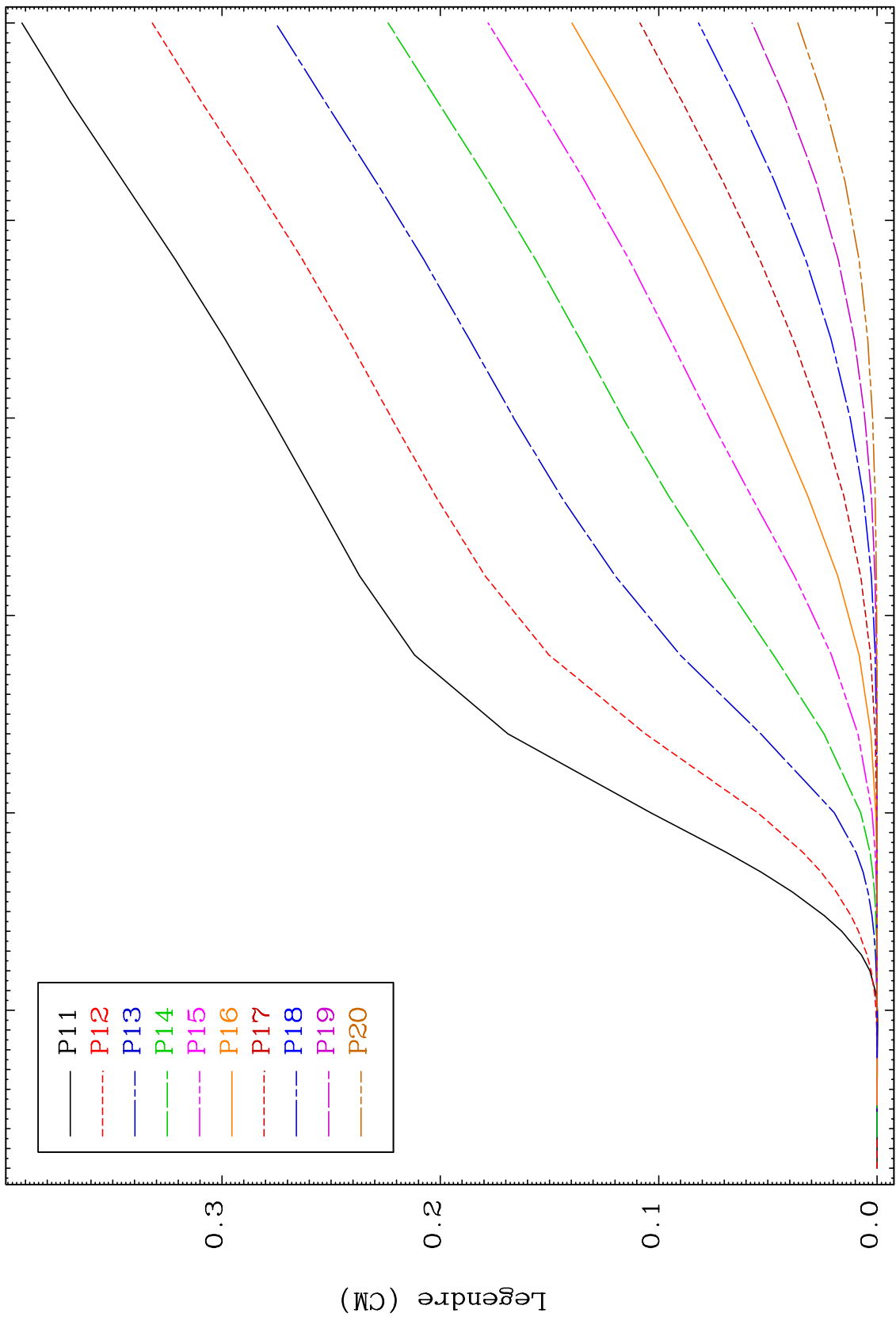
Incident Energy (MeV)

93-Np-241

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Elastic Legendre Coefficients

<sup>93</sup>Np-241



20

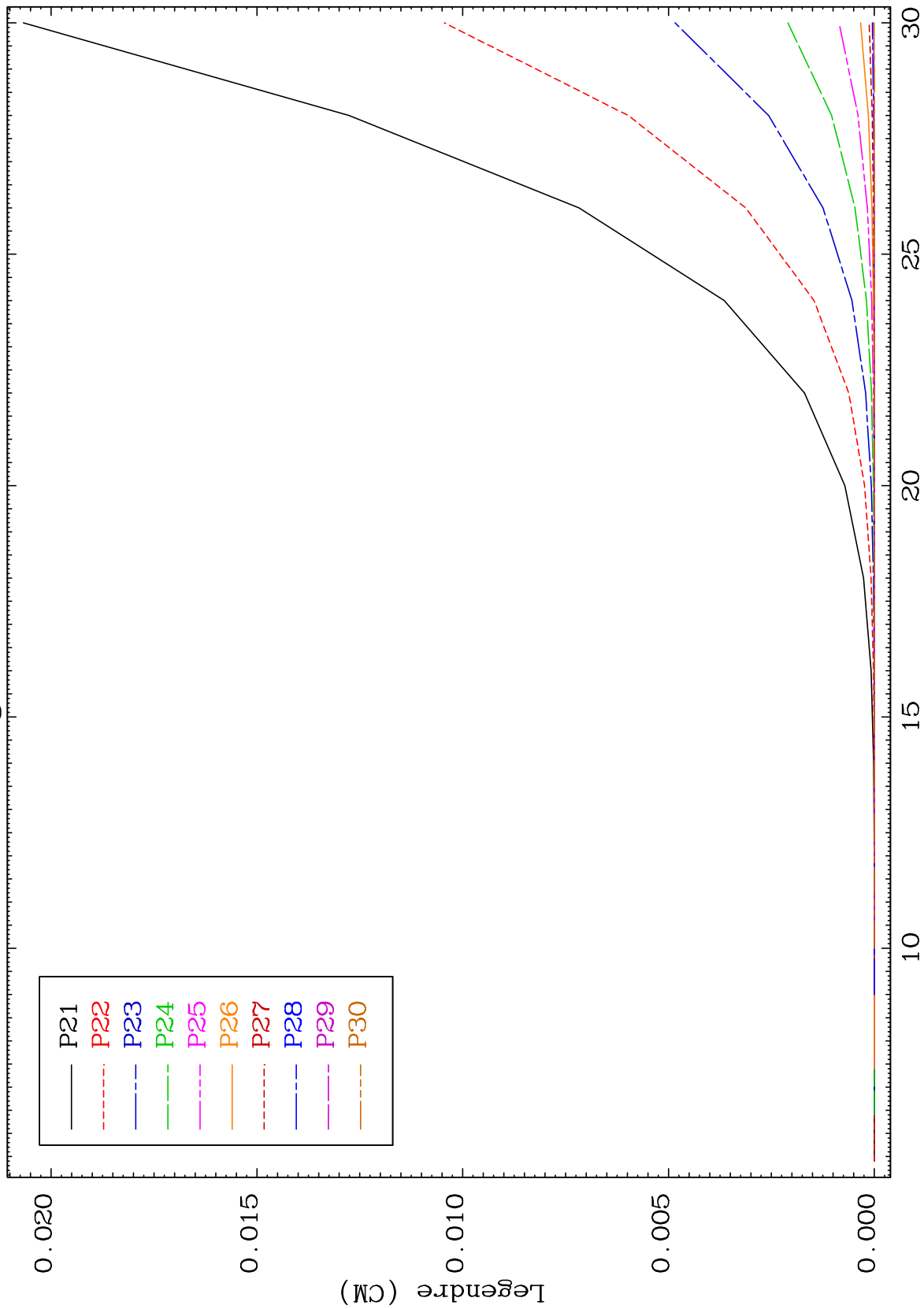
Incident Energy (MeV)

<sup>93</sup>Np-241

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### Elastic Legendre Coefficients

93-Np-241



21

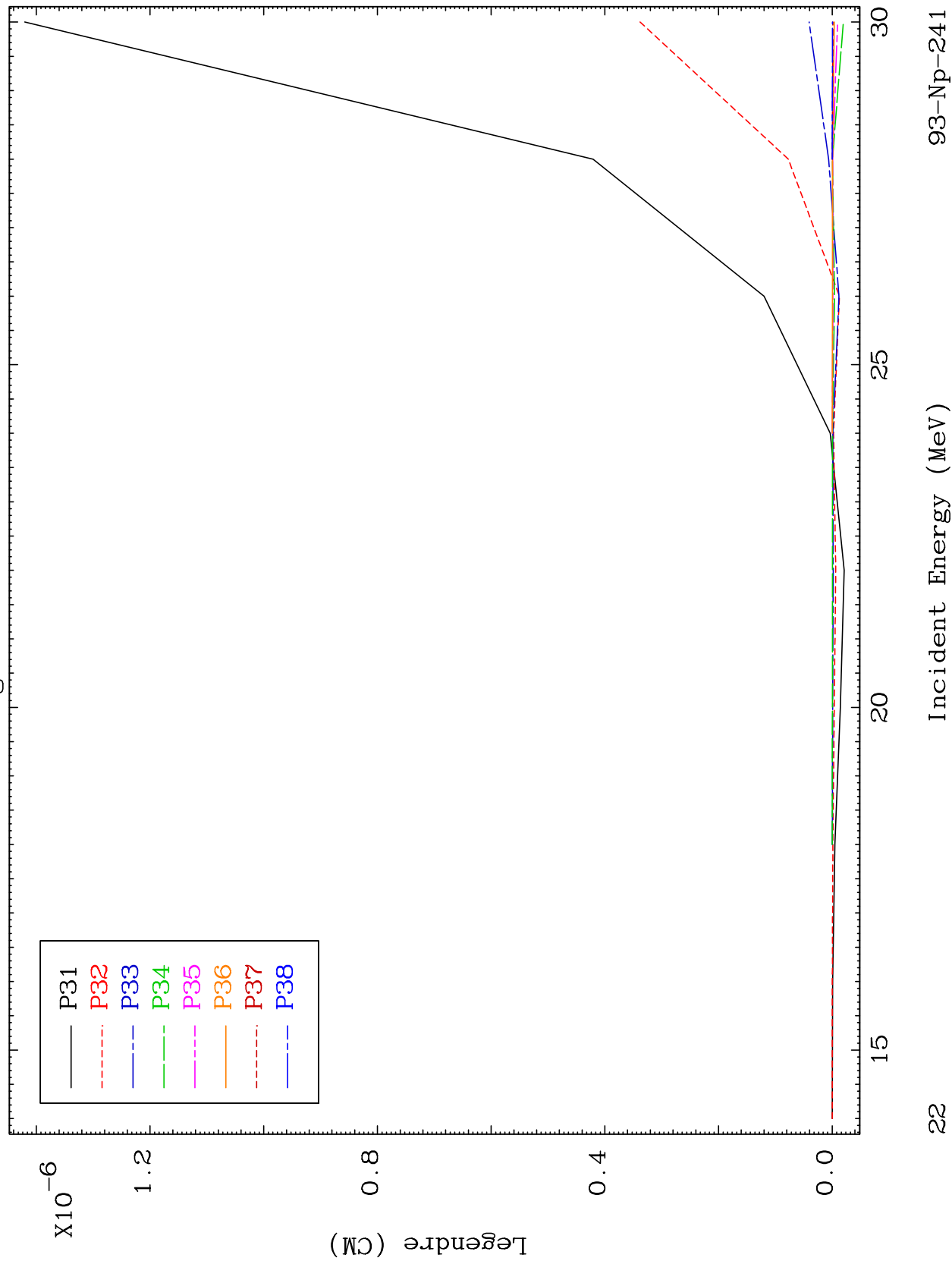
93-Np-241

Incident Energy (MeV)

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Elastic Legendre Coefficients

93-Np-241



22

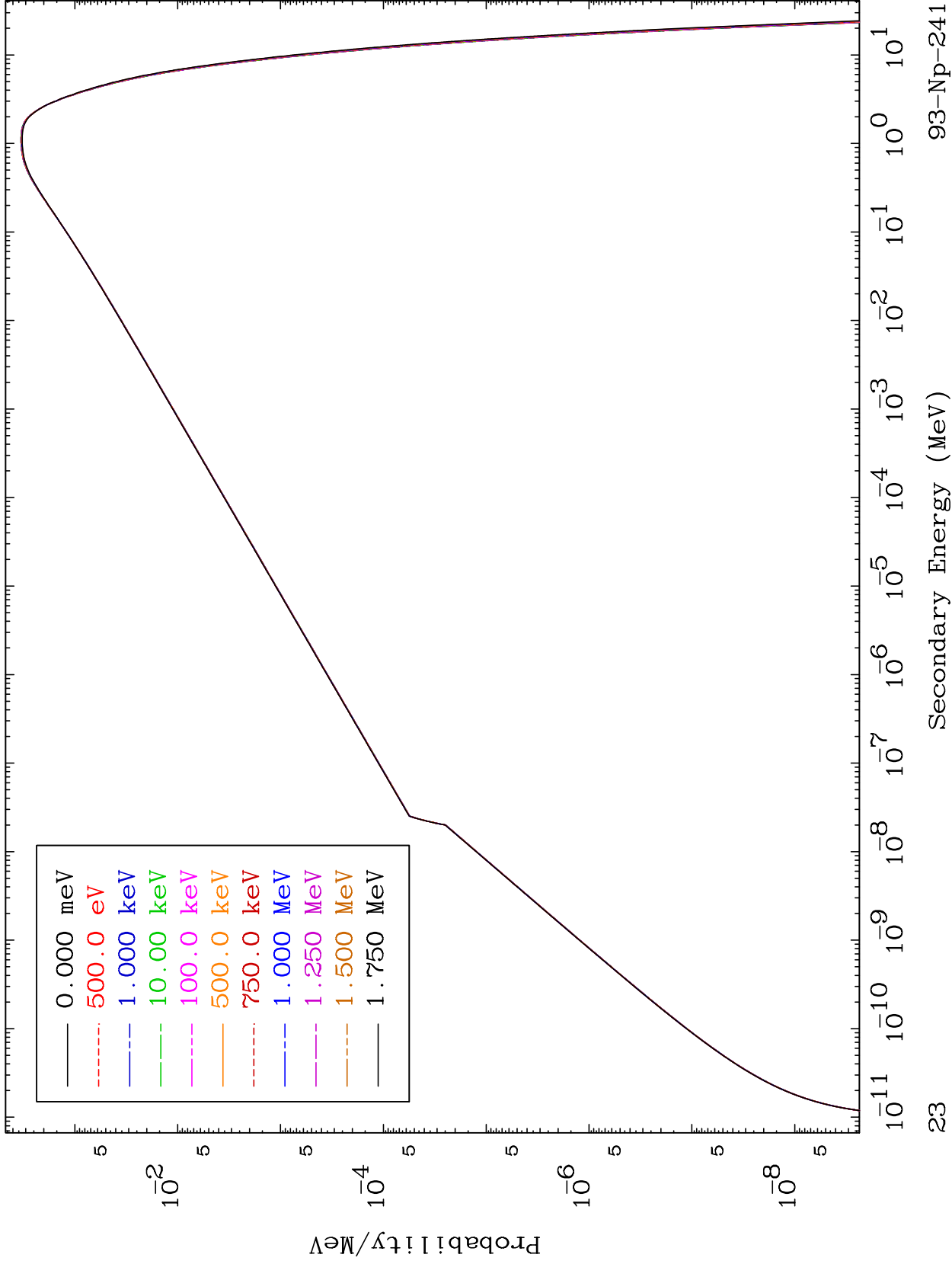
93-Np-241

Incident Energy (MeV)

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### Fission Energy Distributions

<sup>93</sup>Np-241



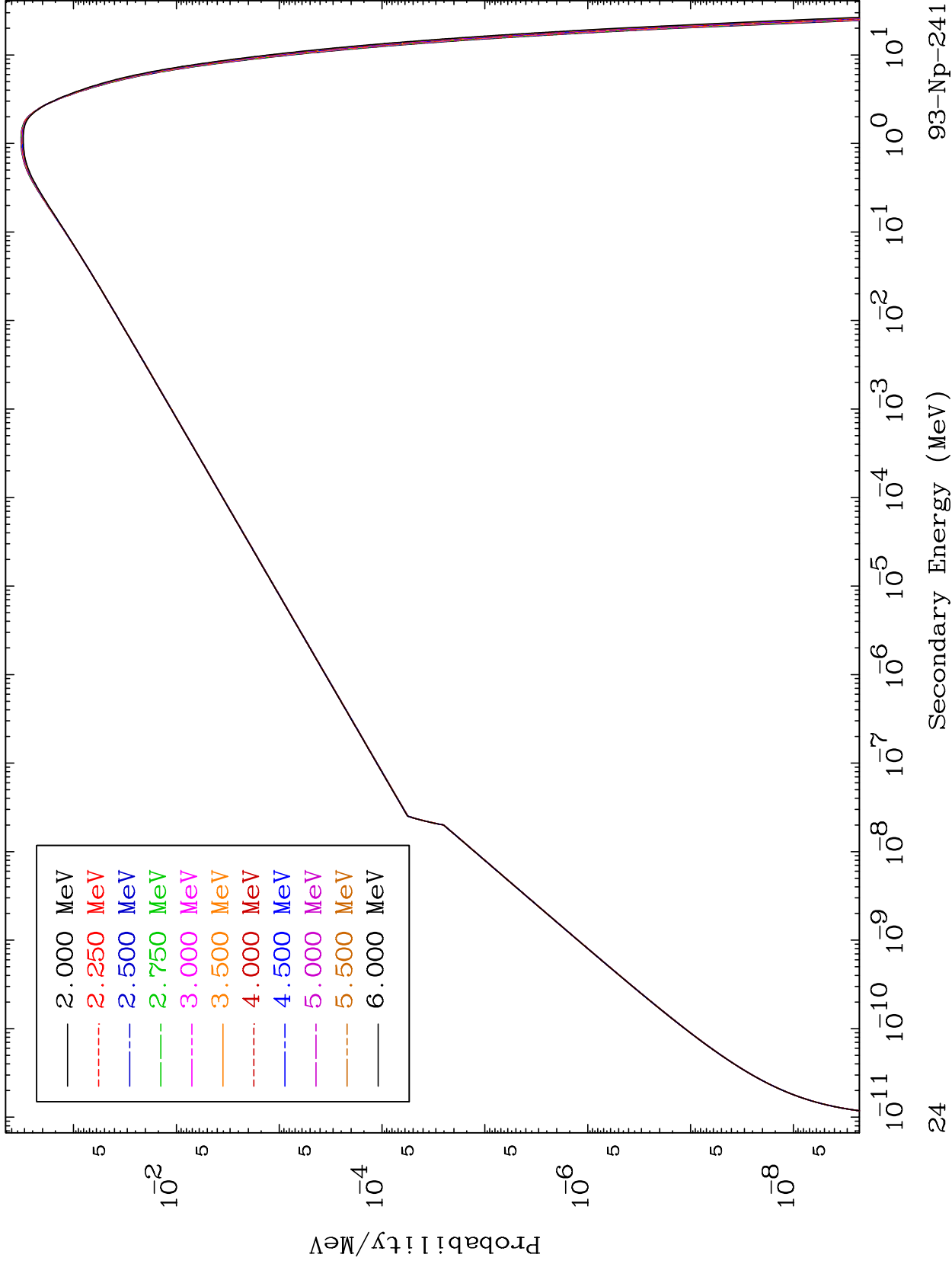
<sup>93</sup>Np-241



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### Fission Energy Distributions

93-Np-241



93-Np-241

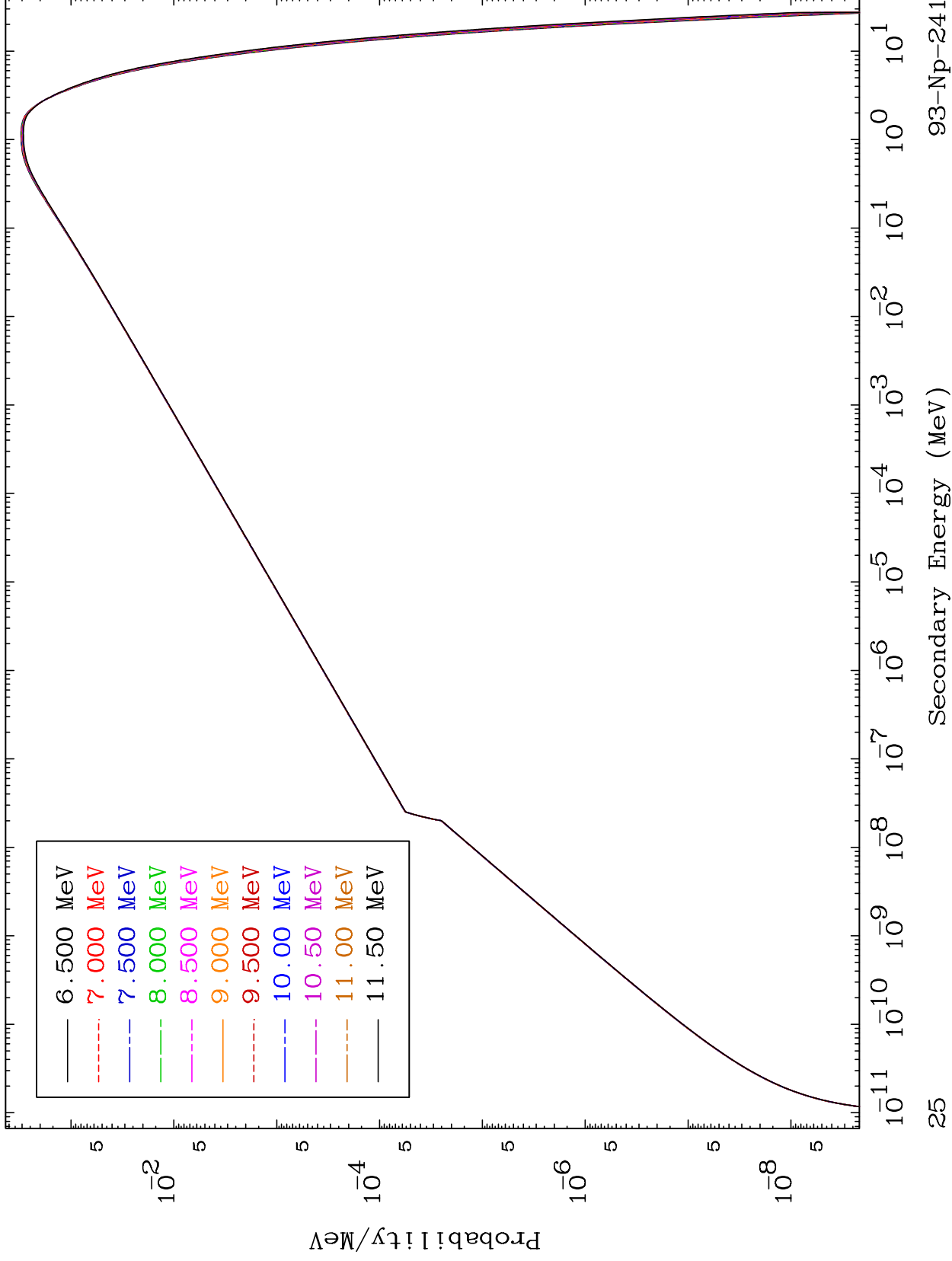
Secondary Energy (MeV)

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### Fission Energy Distributions

93-Np-241



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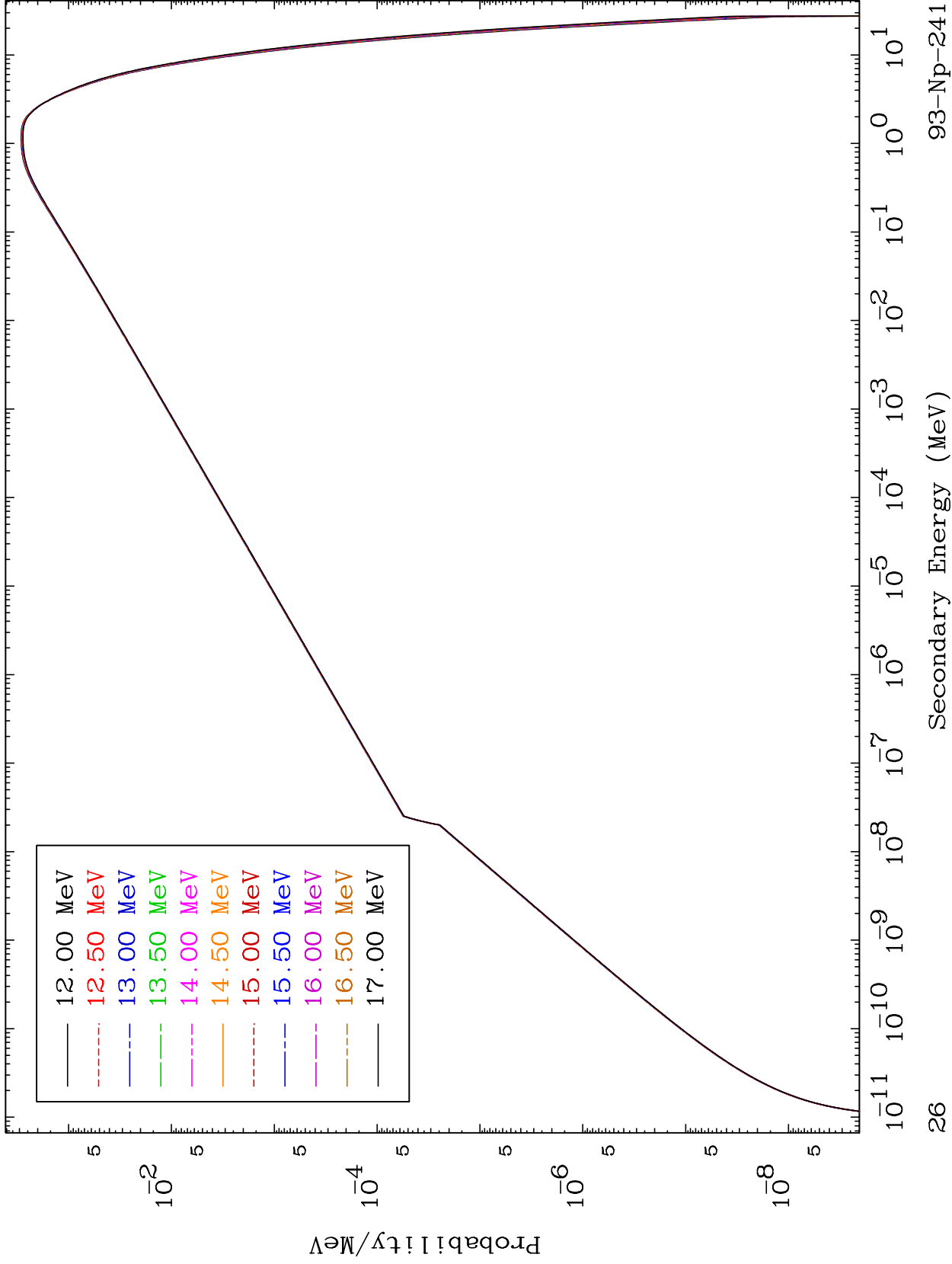
Secondary Energy (MeV)

93-Np-241

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Fission Energy Distributions

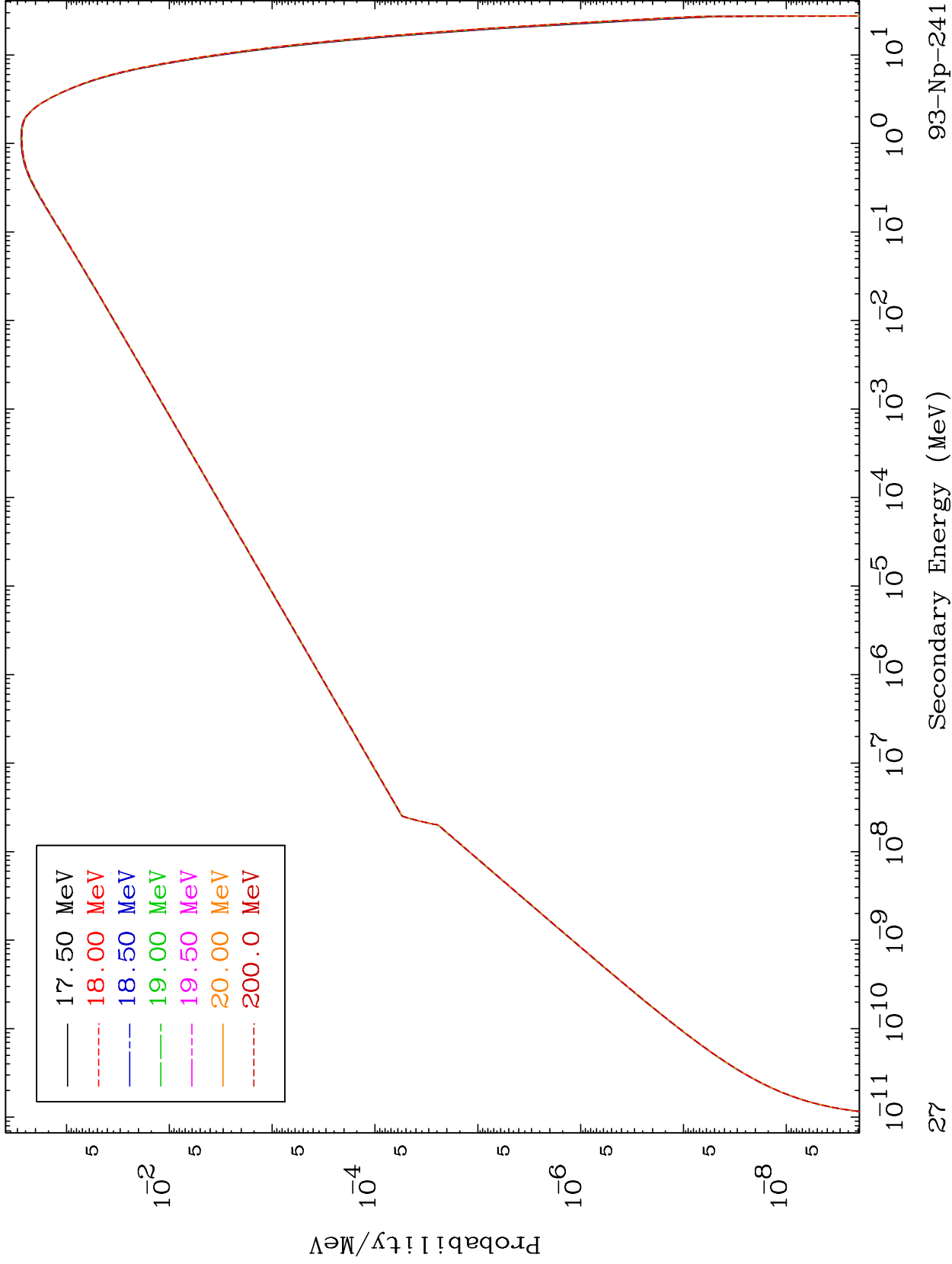
93-Np-241



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### Fission Energy Distributions

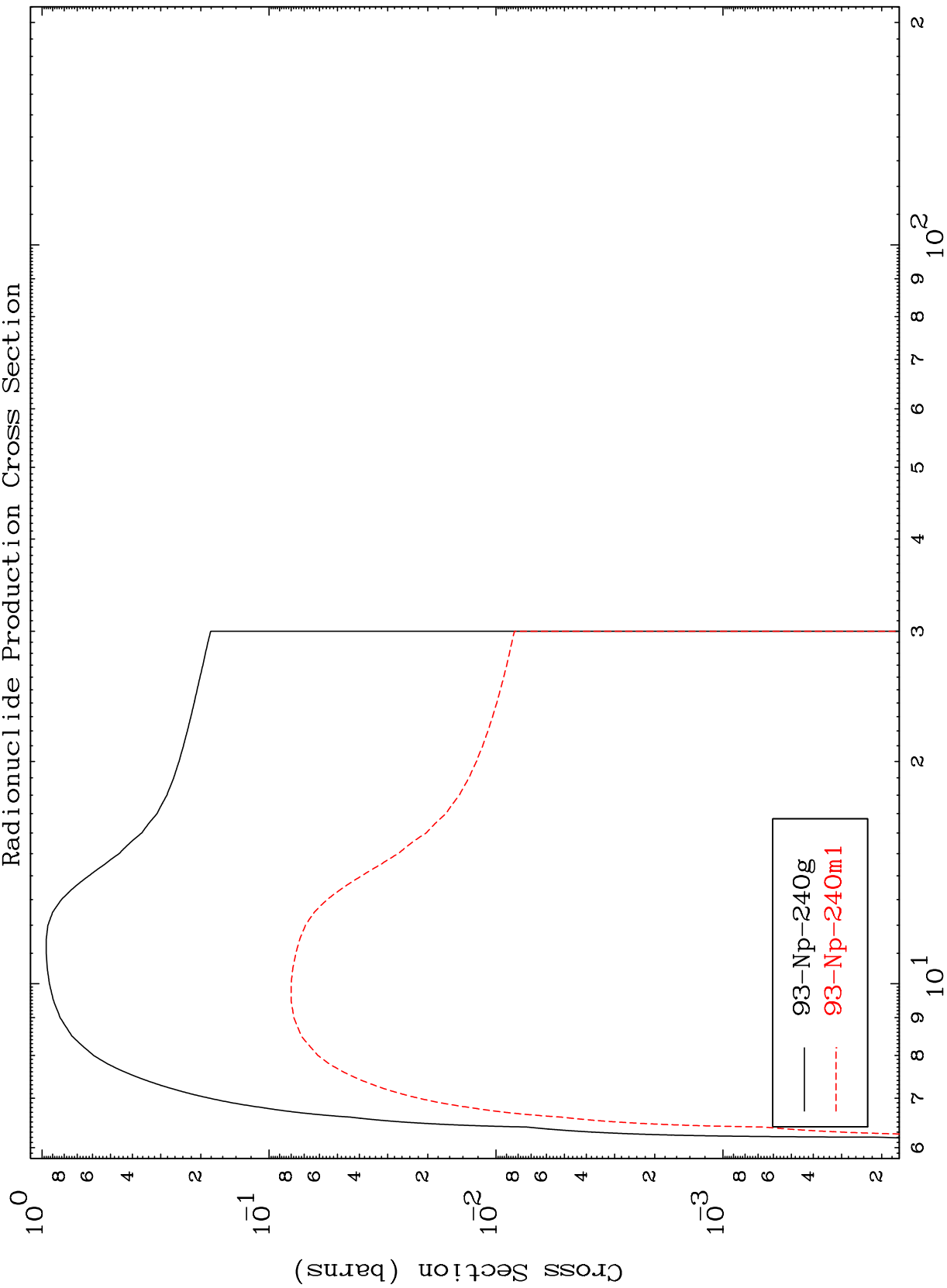
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93-Np-241

Radionuclide Production Cross Section (n,2n)



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Incident Energy (MeV)

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(n,  $\gamma$ )  
Radionuclide Production Cross Section

