

Program EVALPLOT  
(Version 2021-1)

by

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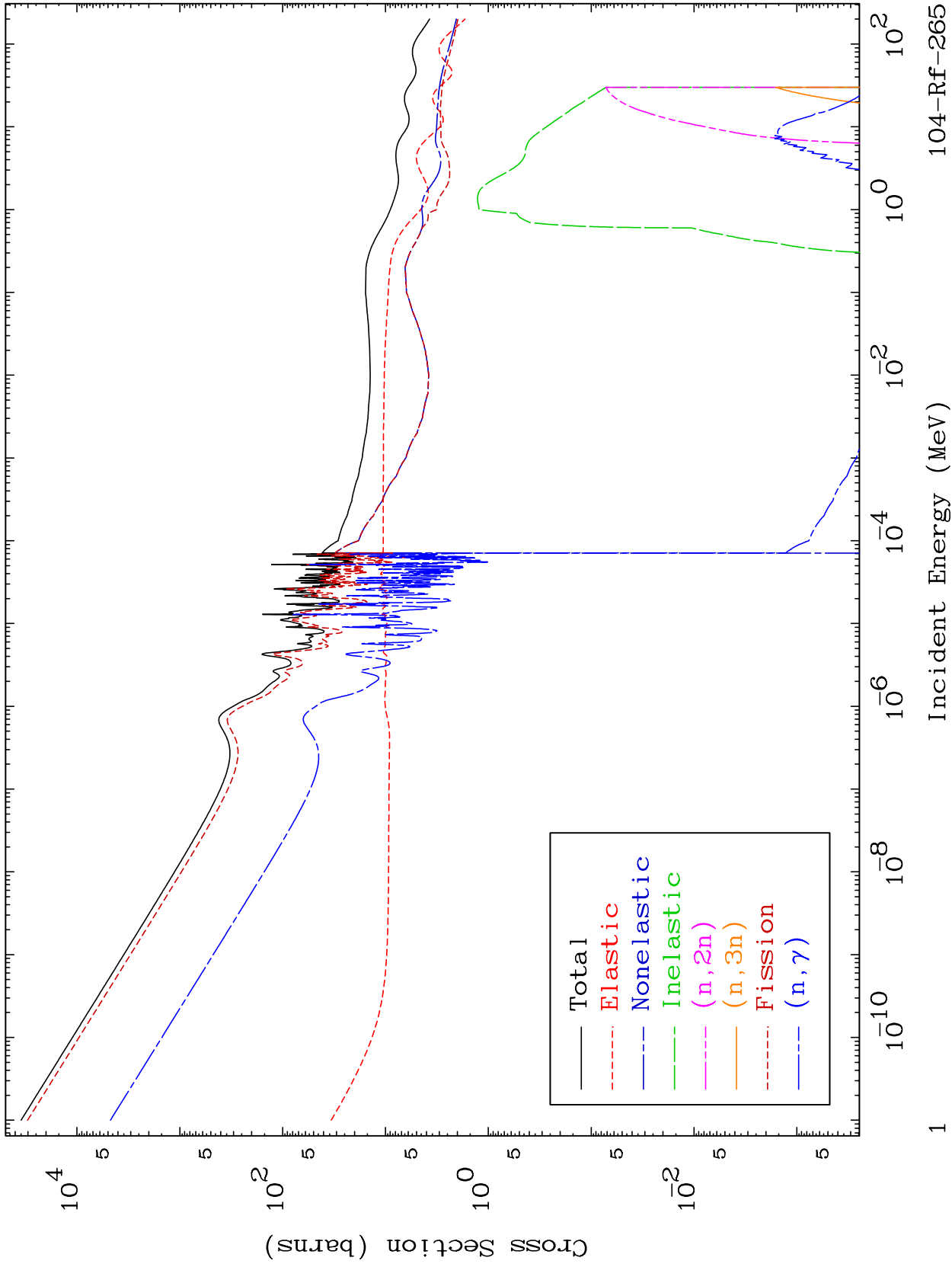
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Press Mouse Button to Start

MAT 465

Neutron Major  
293 Kelvin Cross Sections

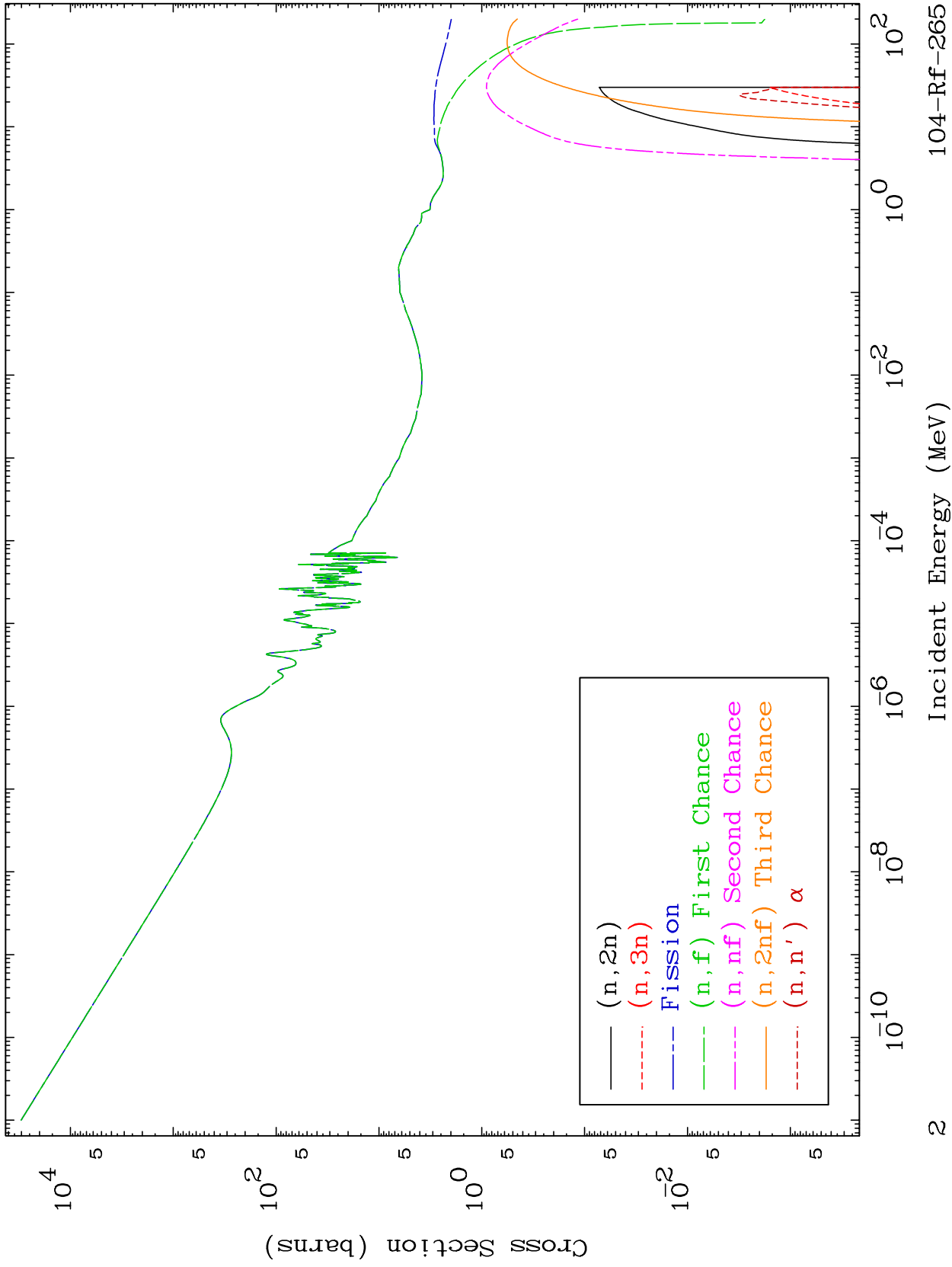
104-Rf-265

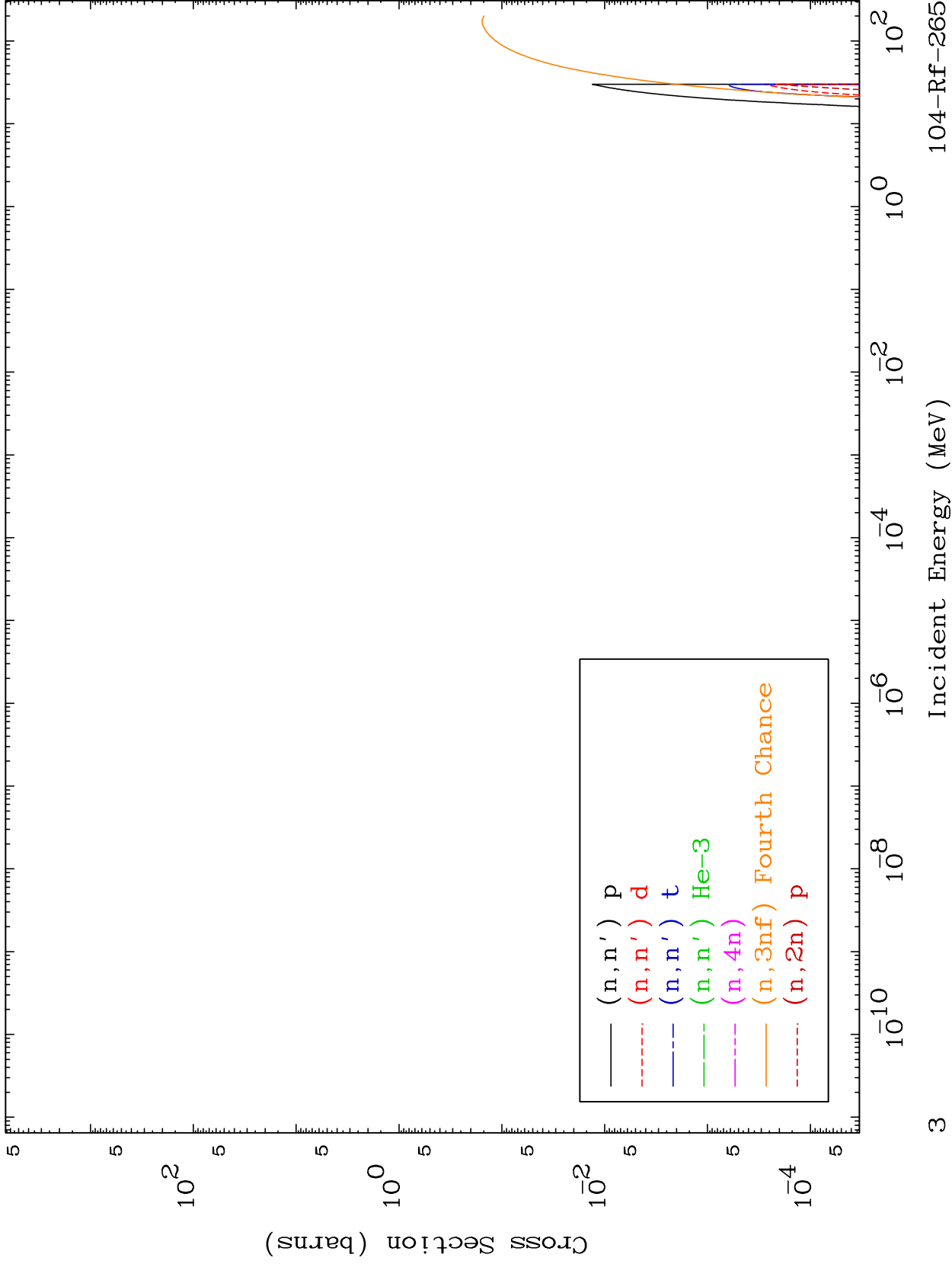


MAT 465

Neutron Absorption  
293 Kelvin Cross Sections

104-Rf-265

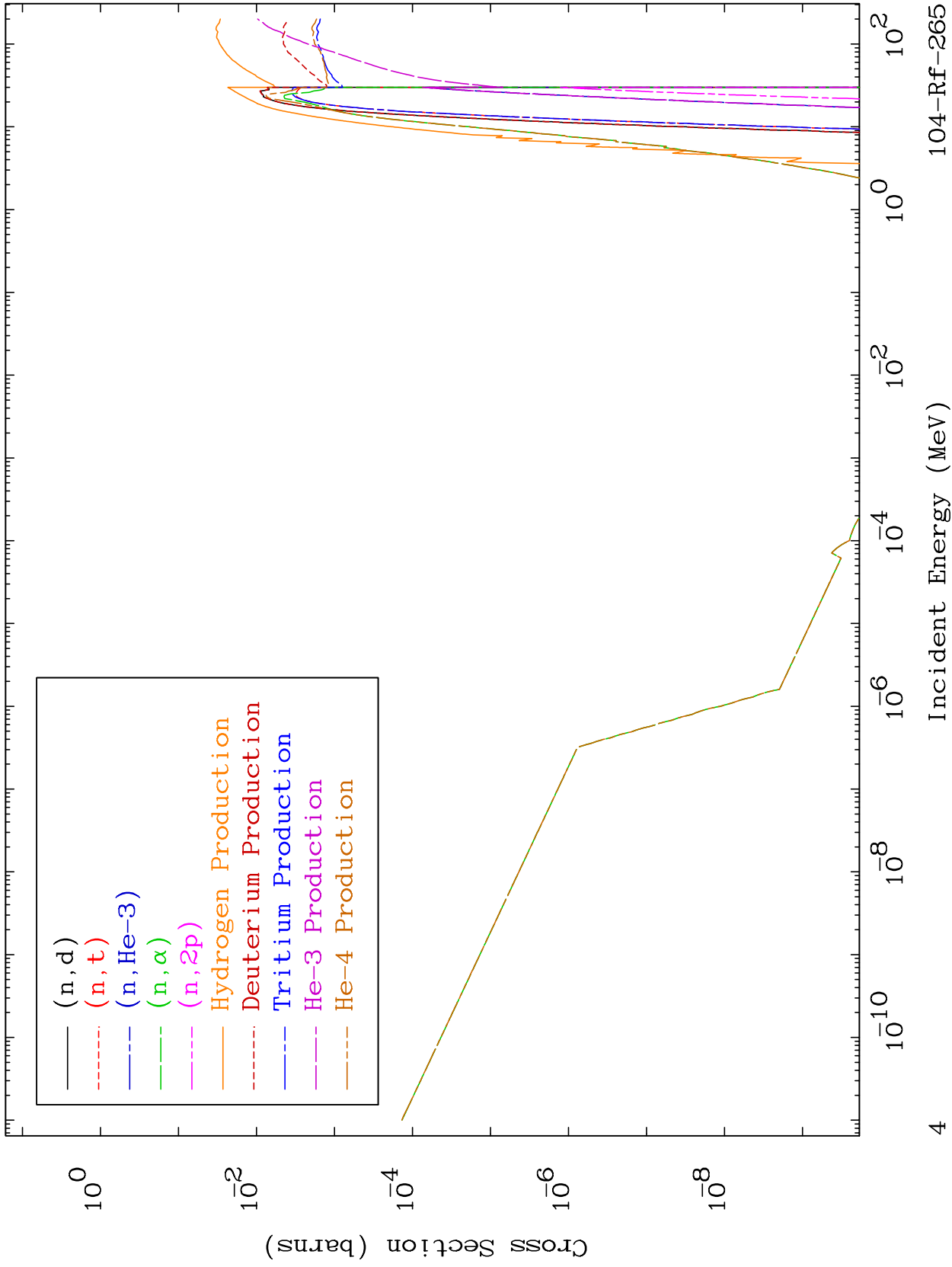




MAT 465

Neutron Absorption  
293 Kelvin Cross Sections

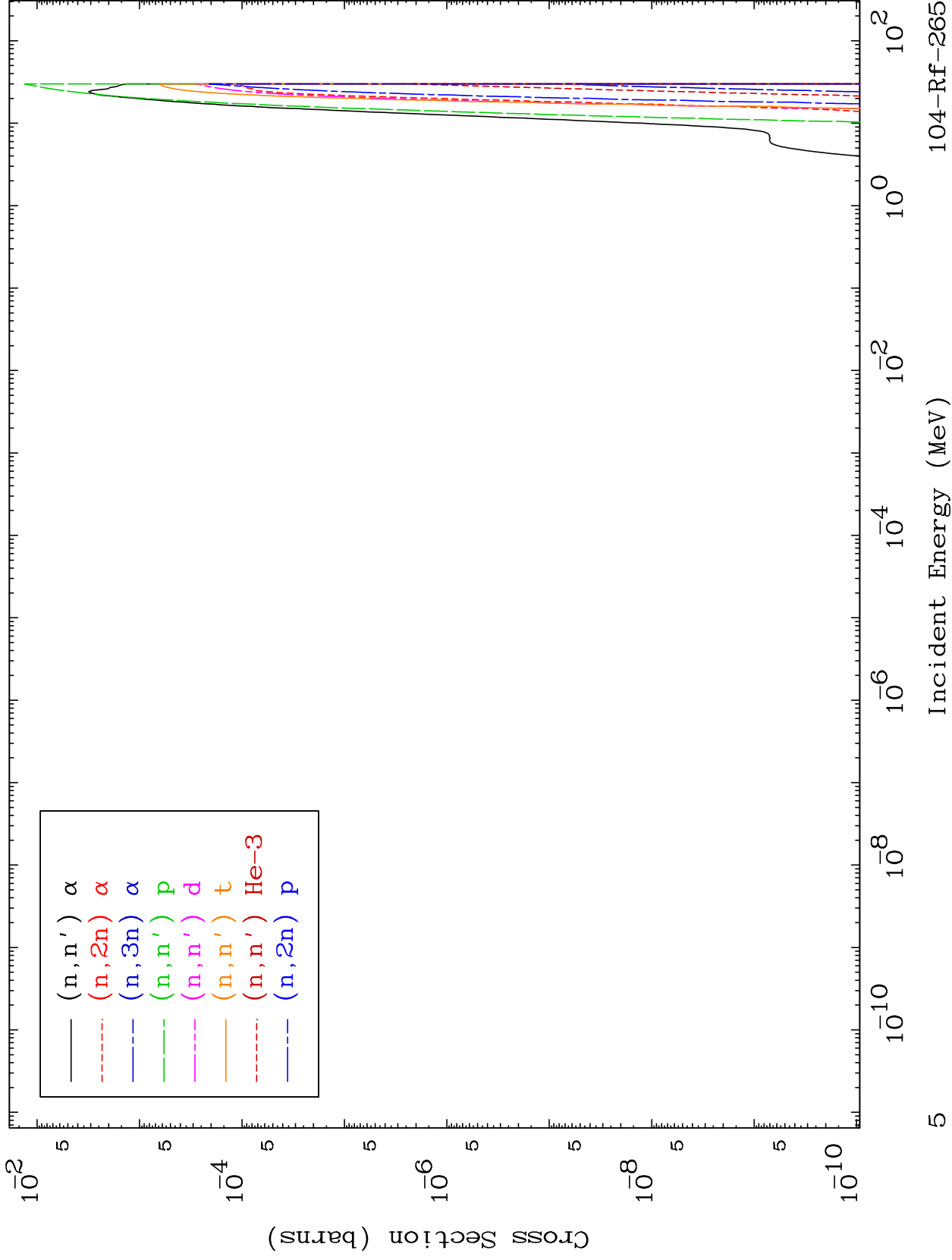
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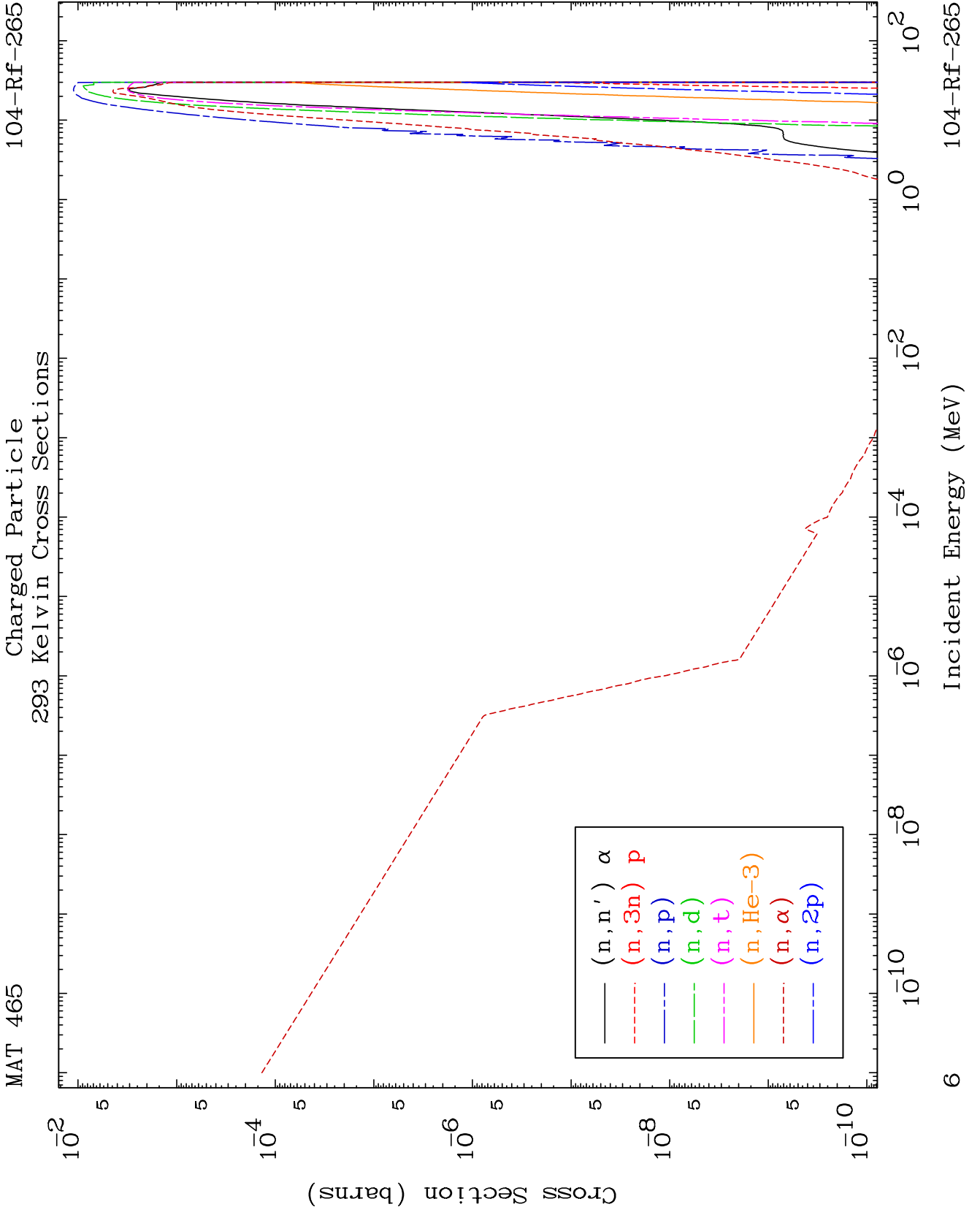


MAT 465

Charged Particle  
293 Kelvin Cross Sections

104-Rf-265

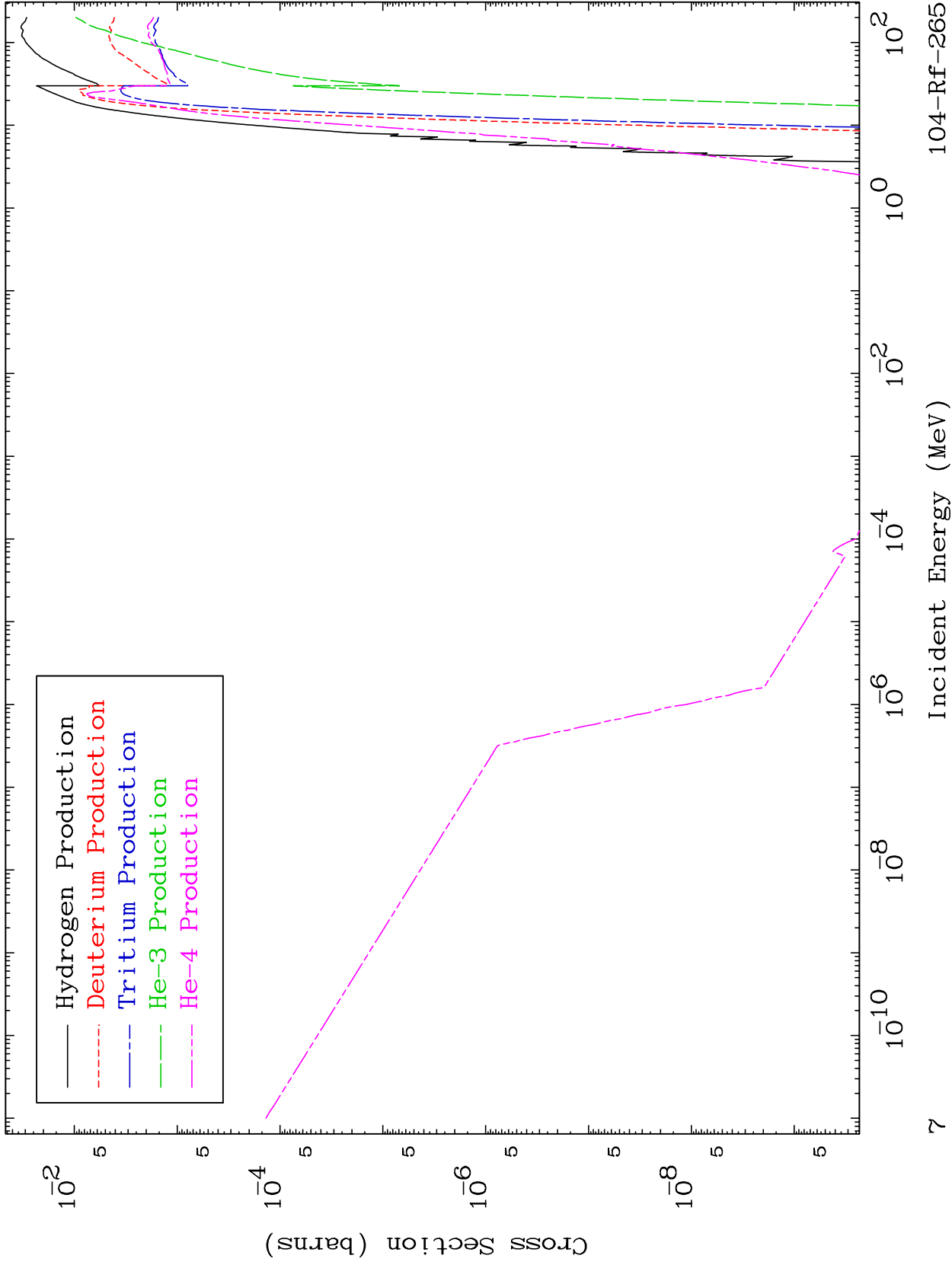




MAT 465

Particle Production  
293 Kelvin Cross Sections

104-Rf-265

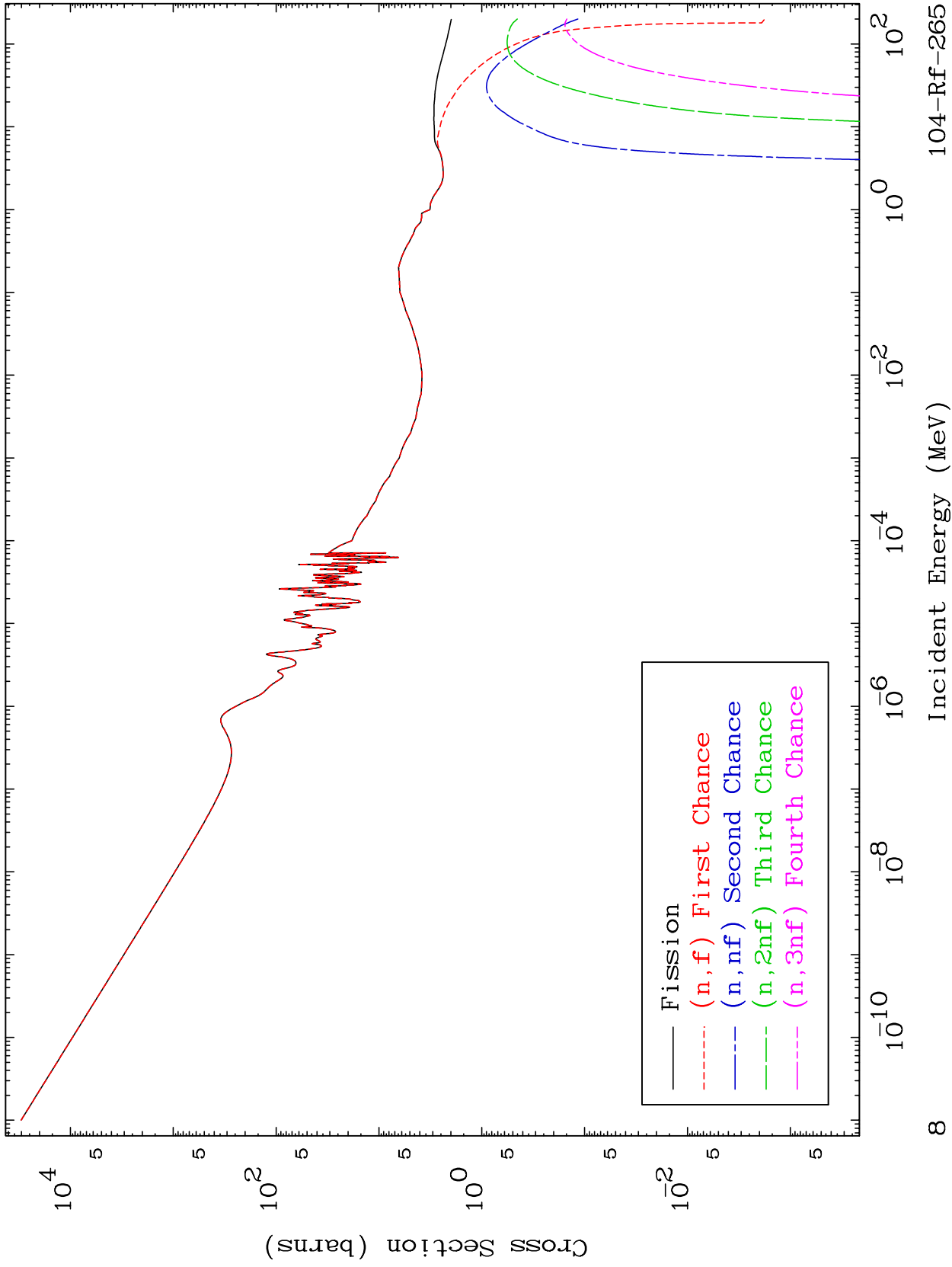




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Fission  
293 Kelvin Cross Sections

104-Rf-265

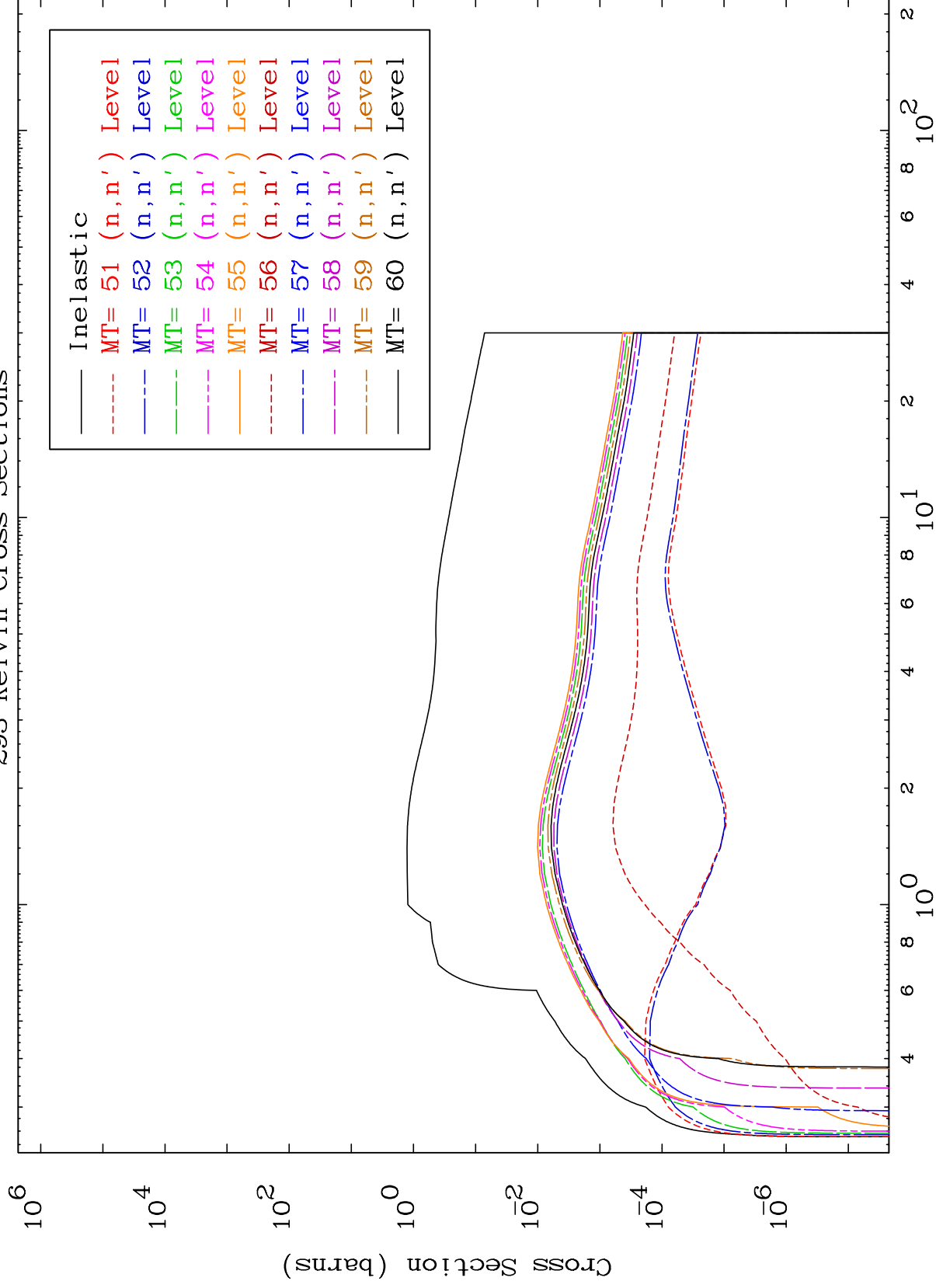


MAT 465

(n,n') Levels

104-Rf-265

293 Kelvin Cross Sections



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Incident Energy (MeV)

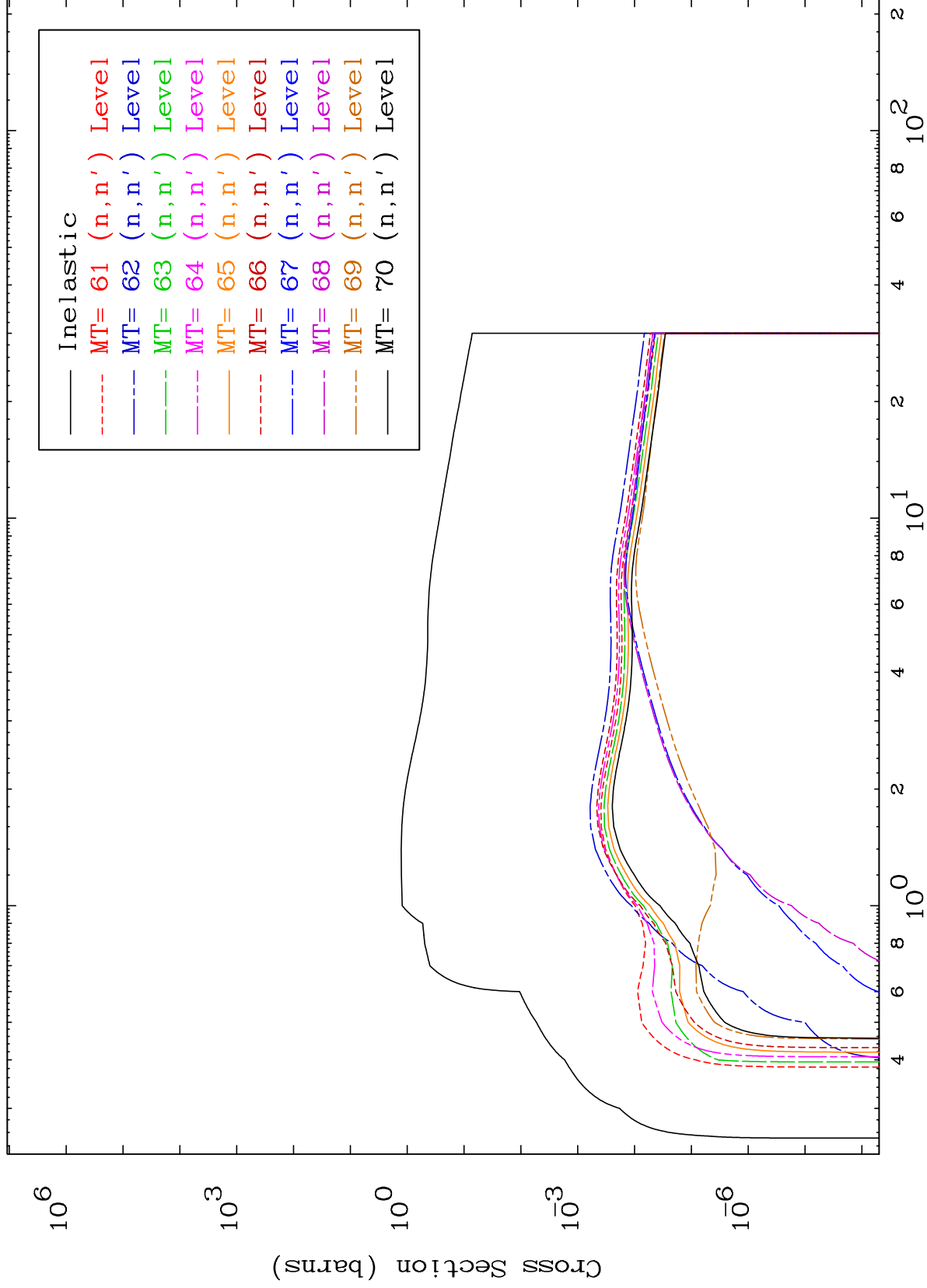
104-Rf-265

MAT 465

(n,n') Levels

104-Rf-265

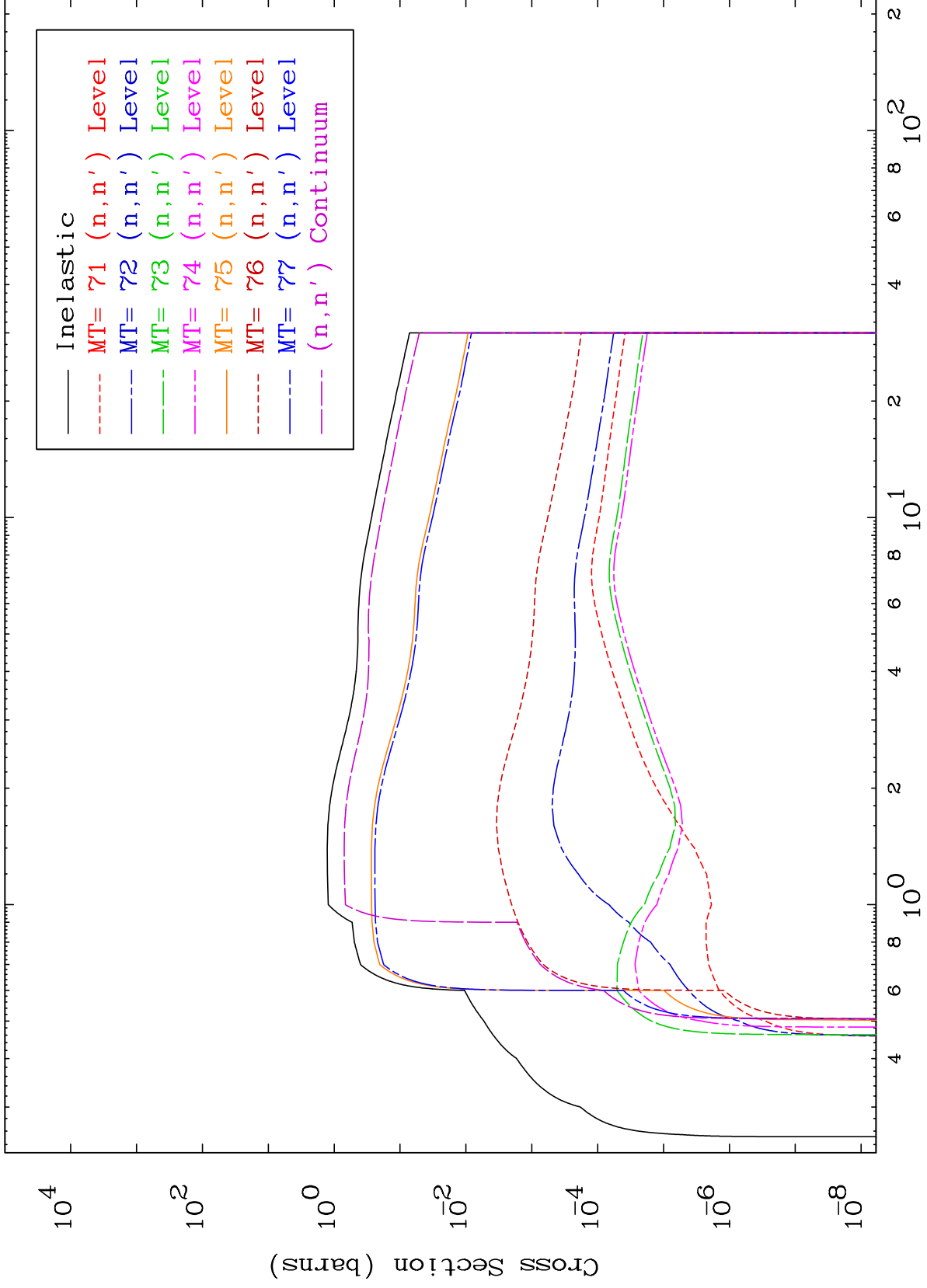
293 Kelvin Cross Sections



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Incident Energy (MeV)

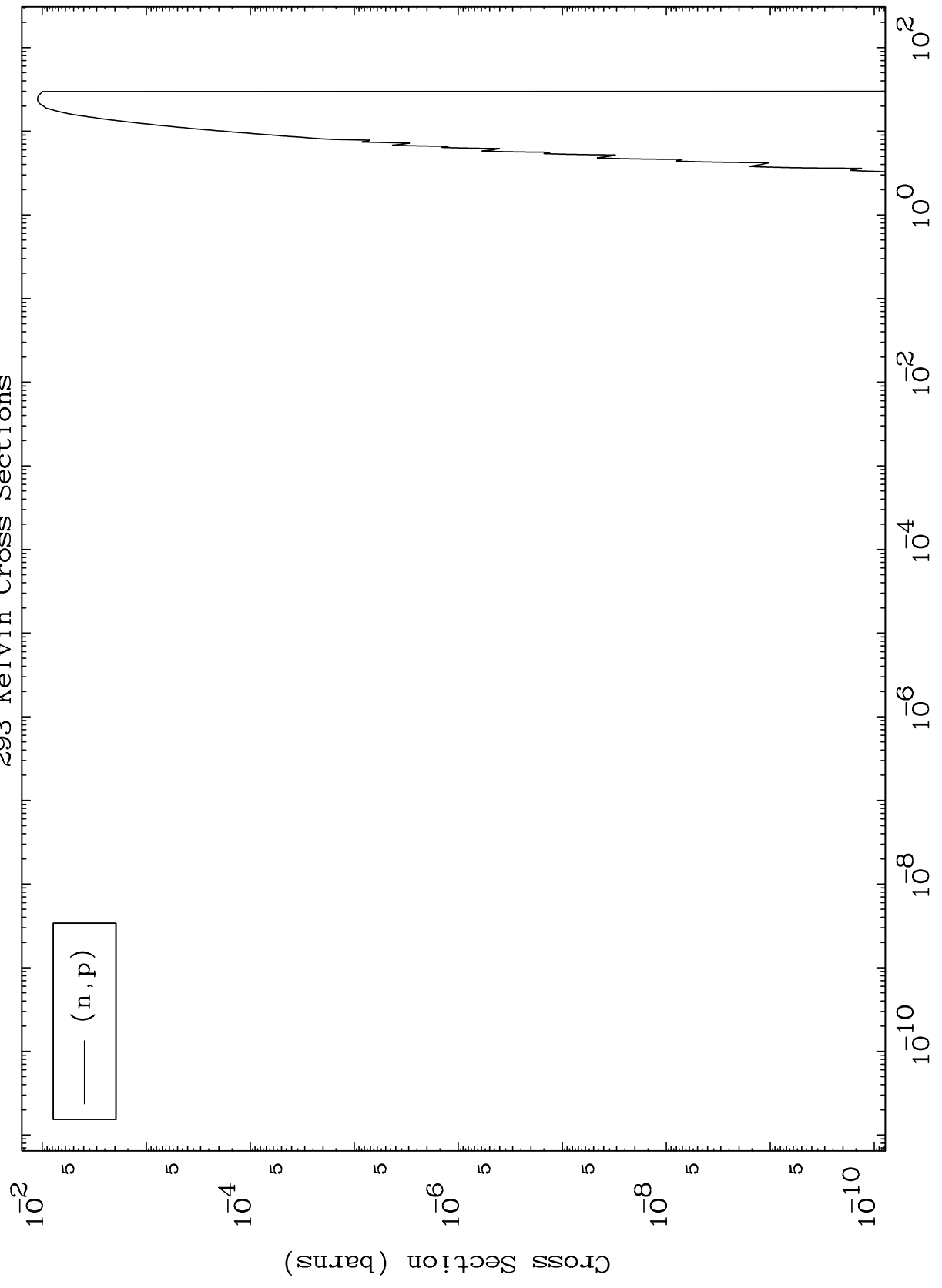
104-Rf-265



MAT 465

(n,p) Levels  
293 Kelvin Cross Sections

104-Rf-265



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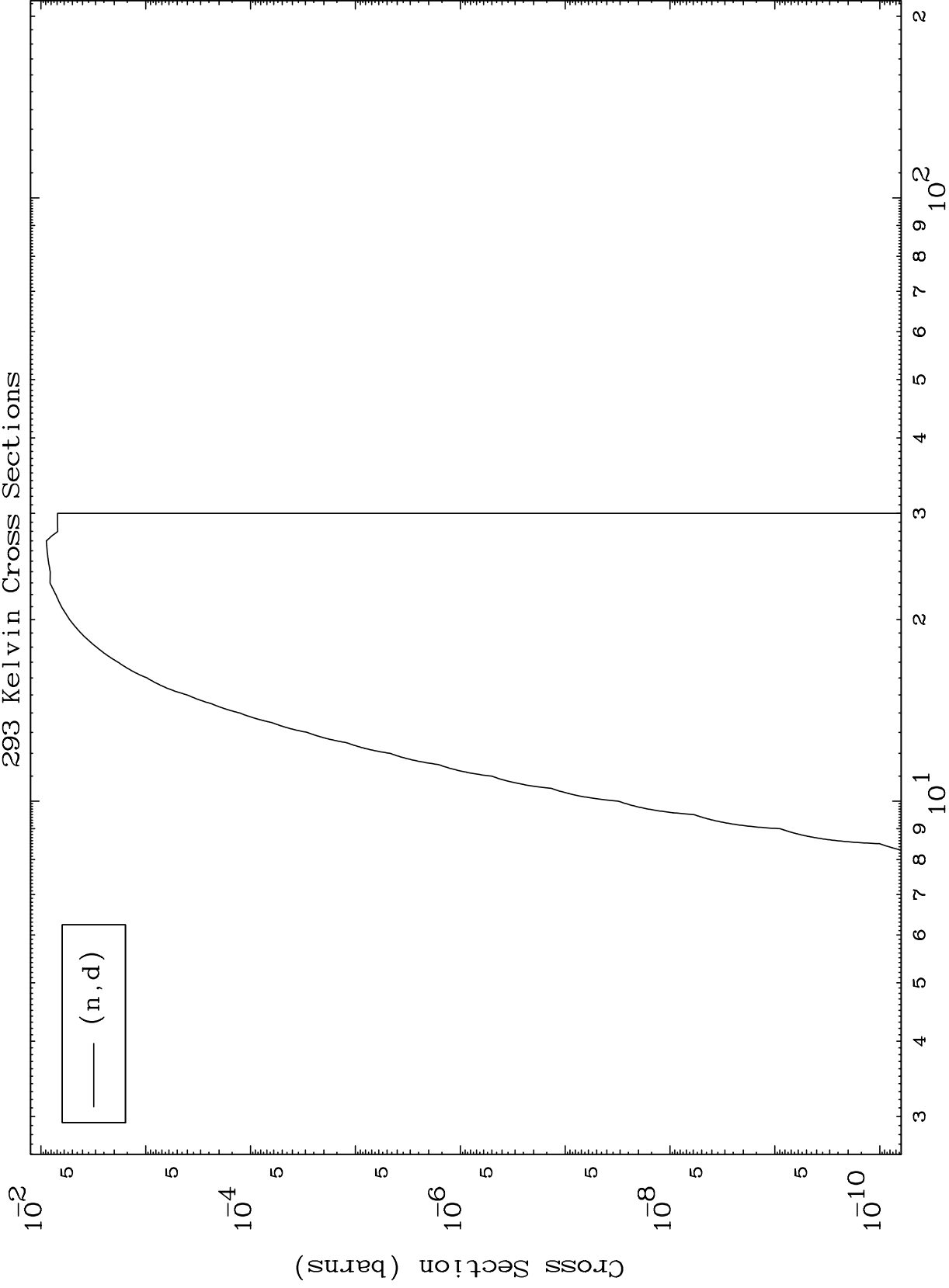
Incident Energy (MeV)

104-Rf-265

MAT 465

(n,d) Levels  
293 Kelvin Cross Sections

104-Rf-265



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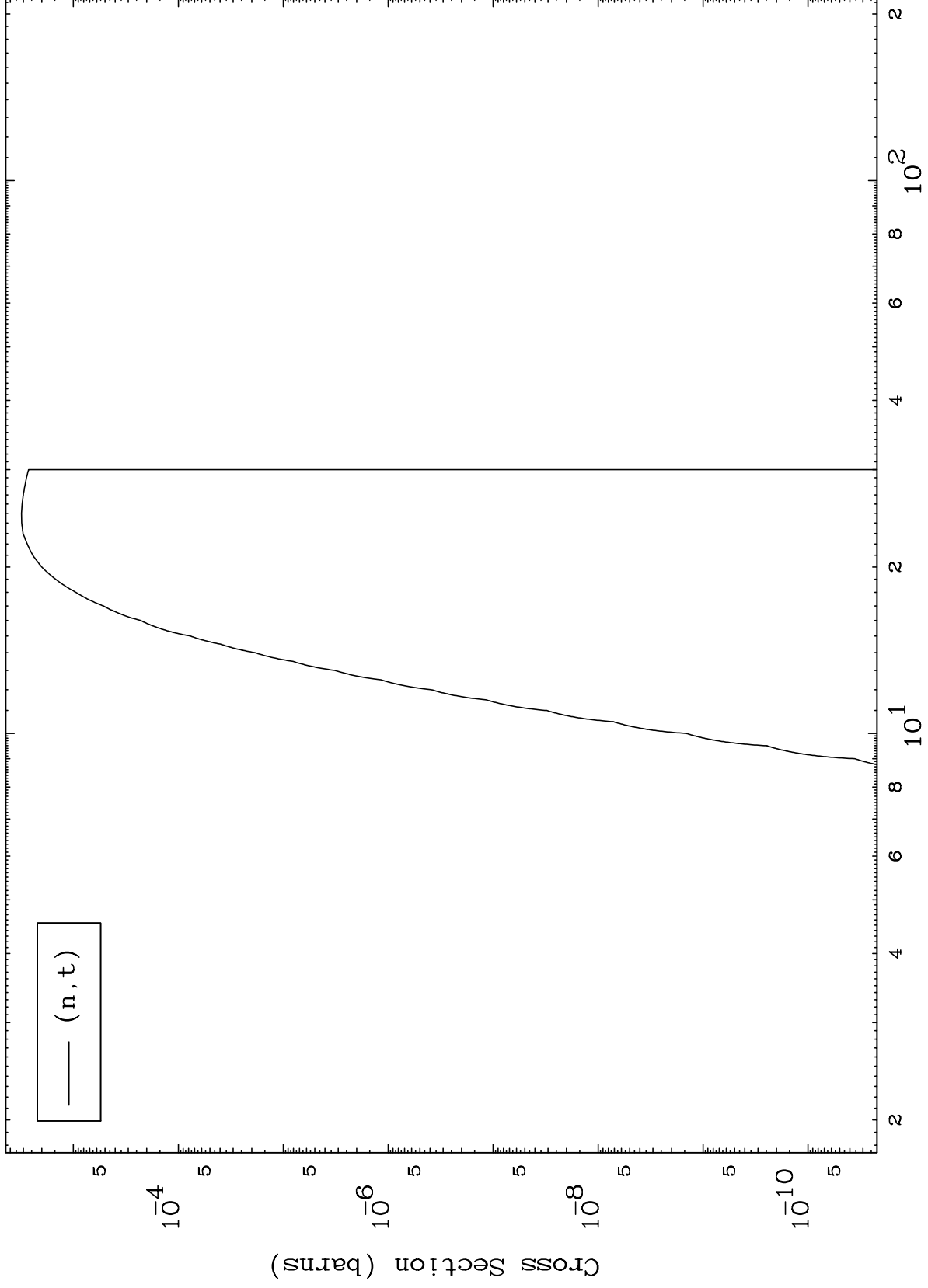
Incident Energy (MeV)

104-Rf-265

MAT 465

(n,t) Levels  
293 Kelvin Cross Sections

104-Rf-265

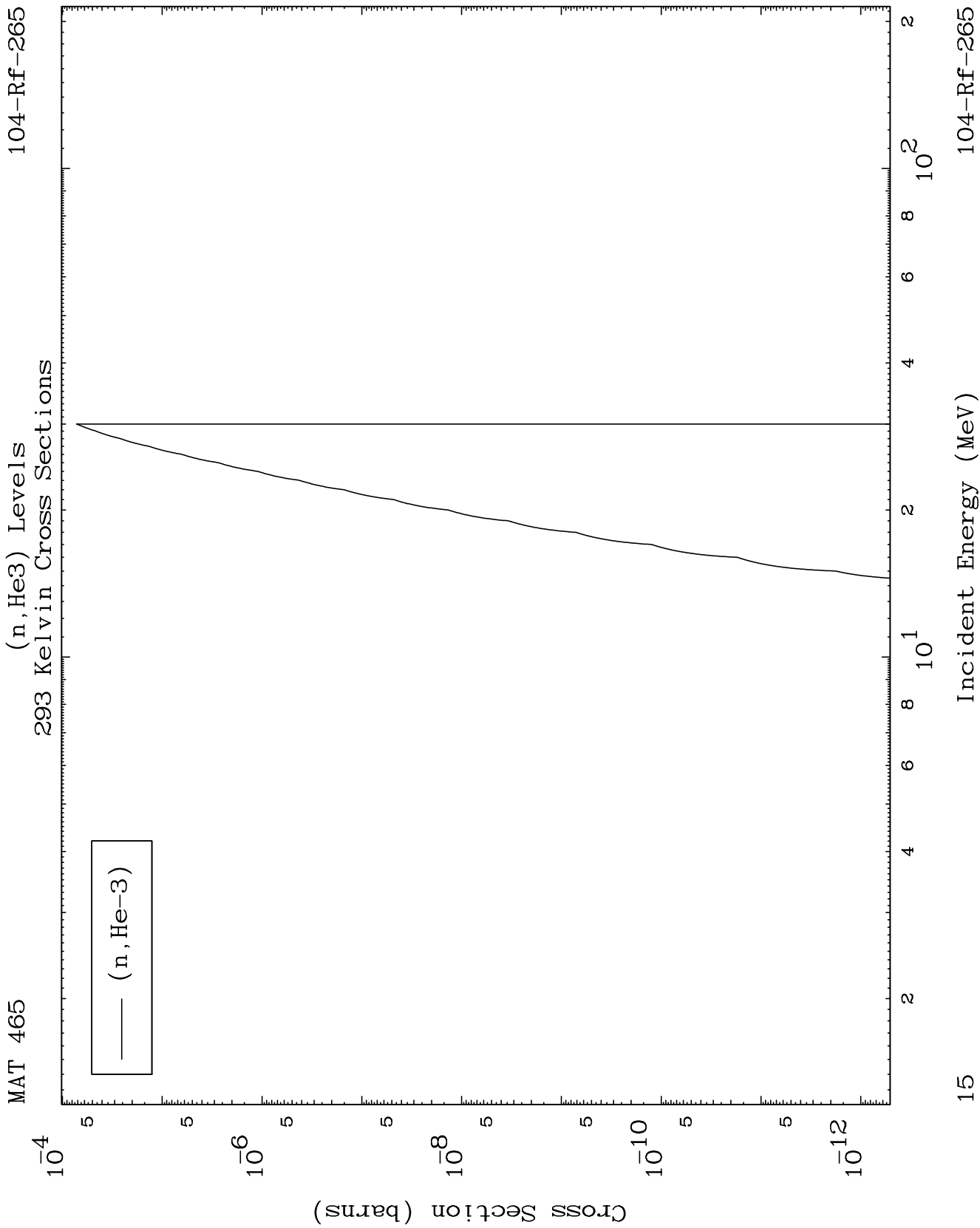


(n,t)

14

Incident Energy (MeV)

104-Rf-265

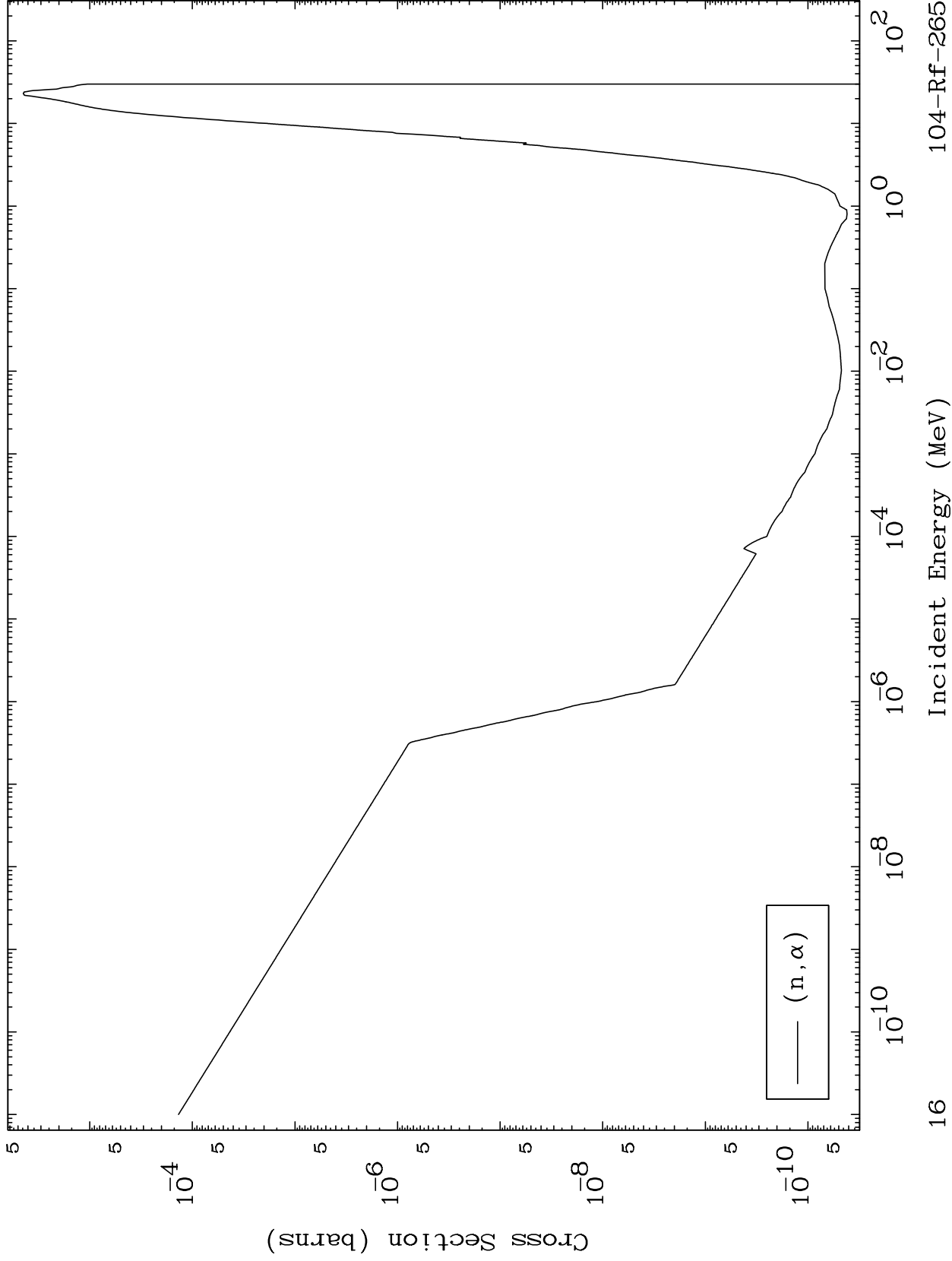




MAT 465

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

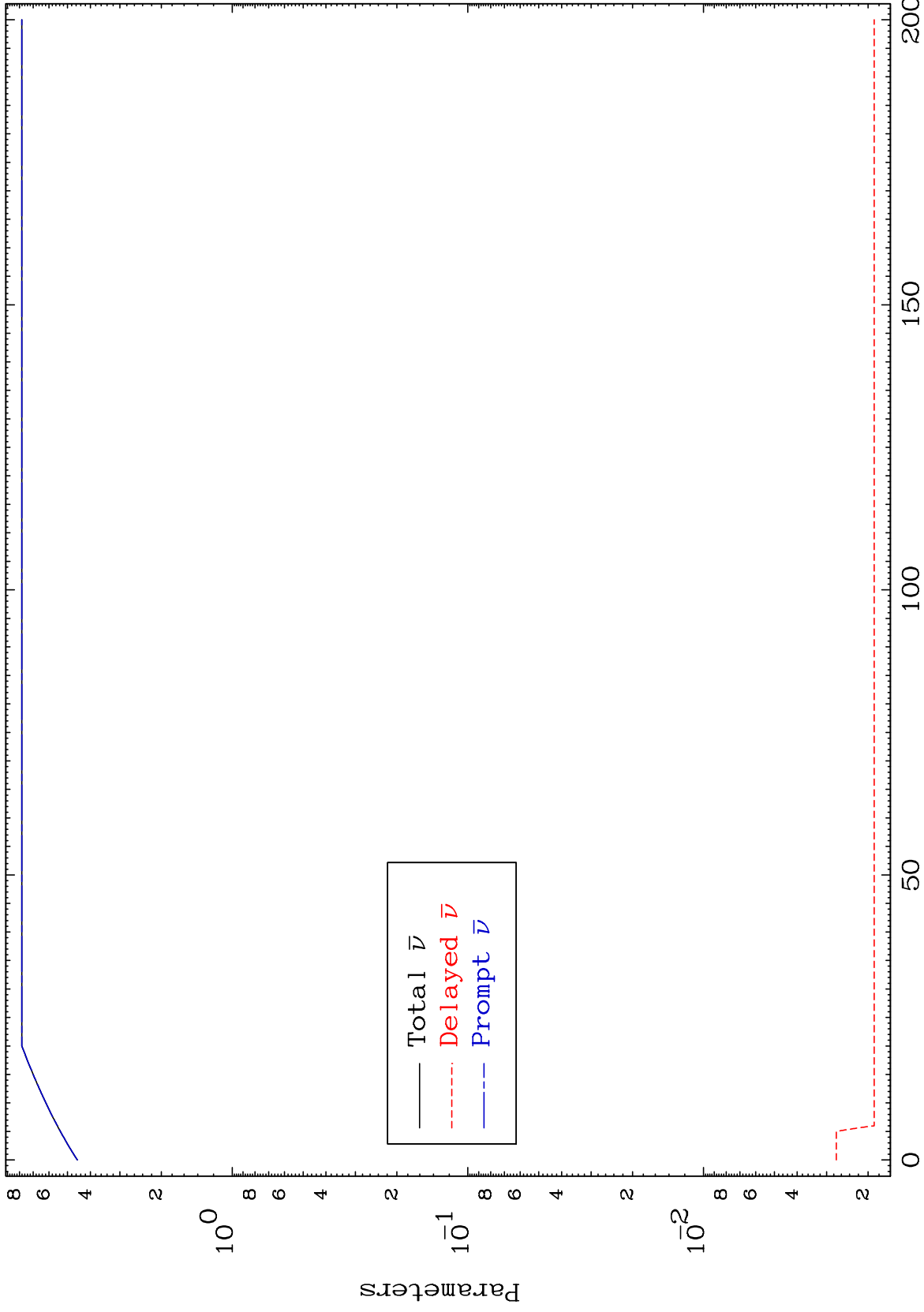
104-Rf-265



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Energy Release  
Parameters

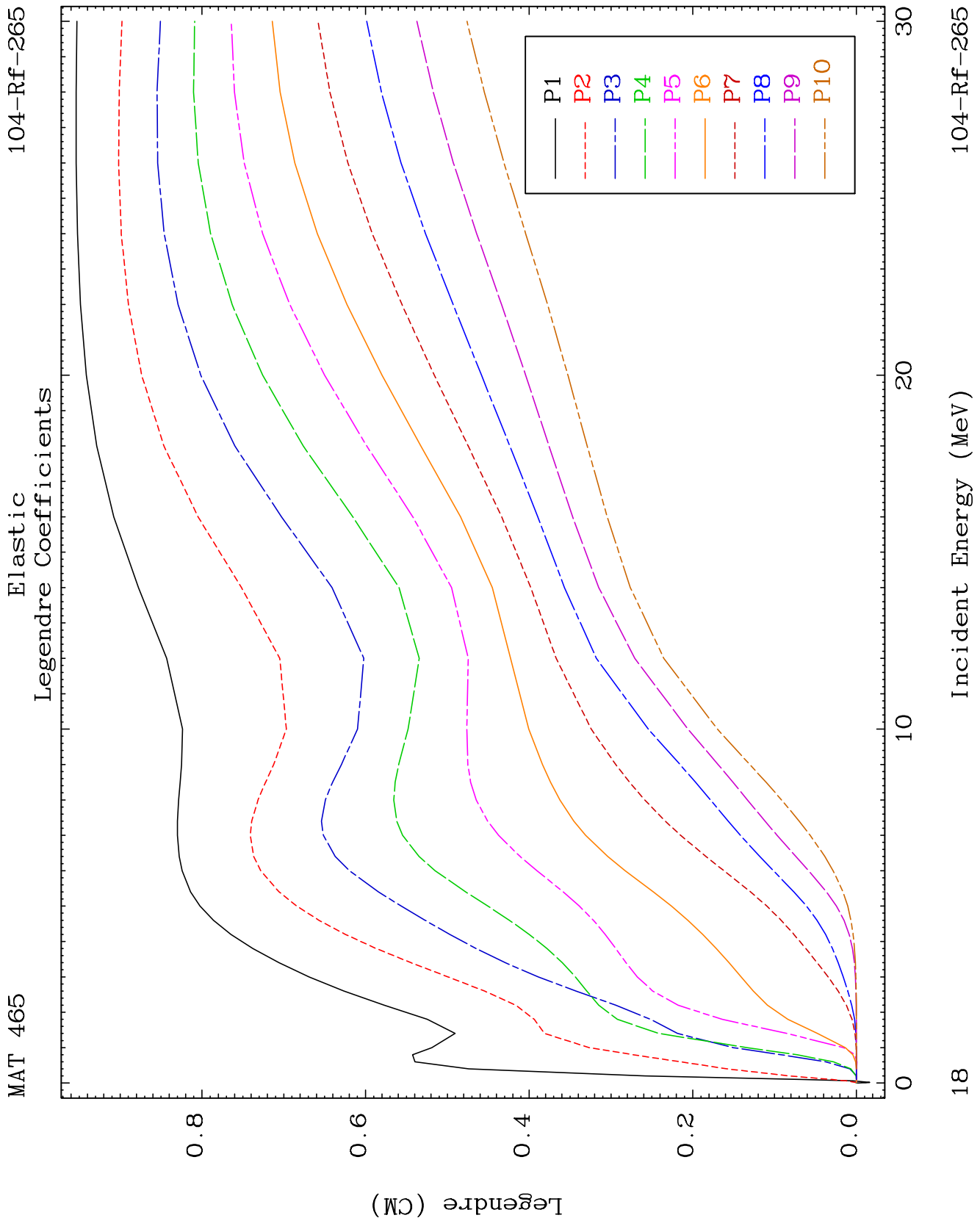
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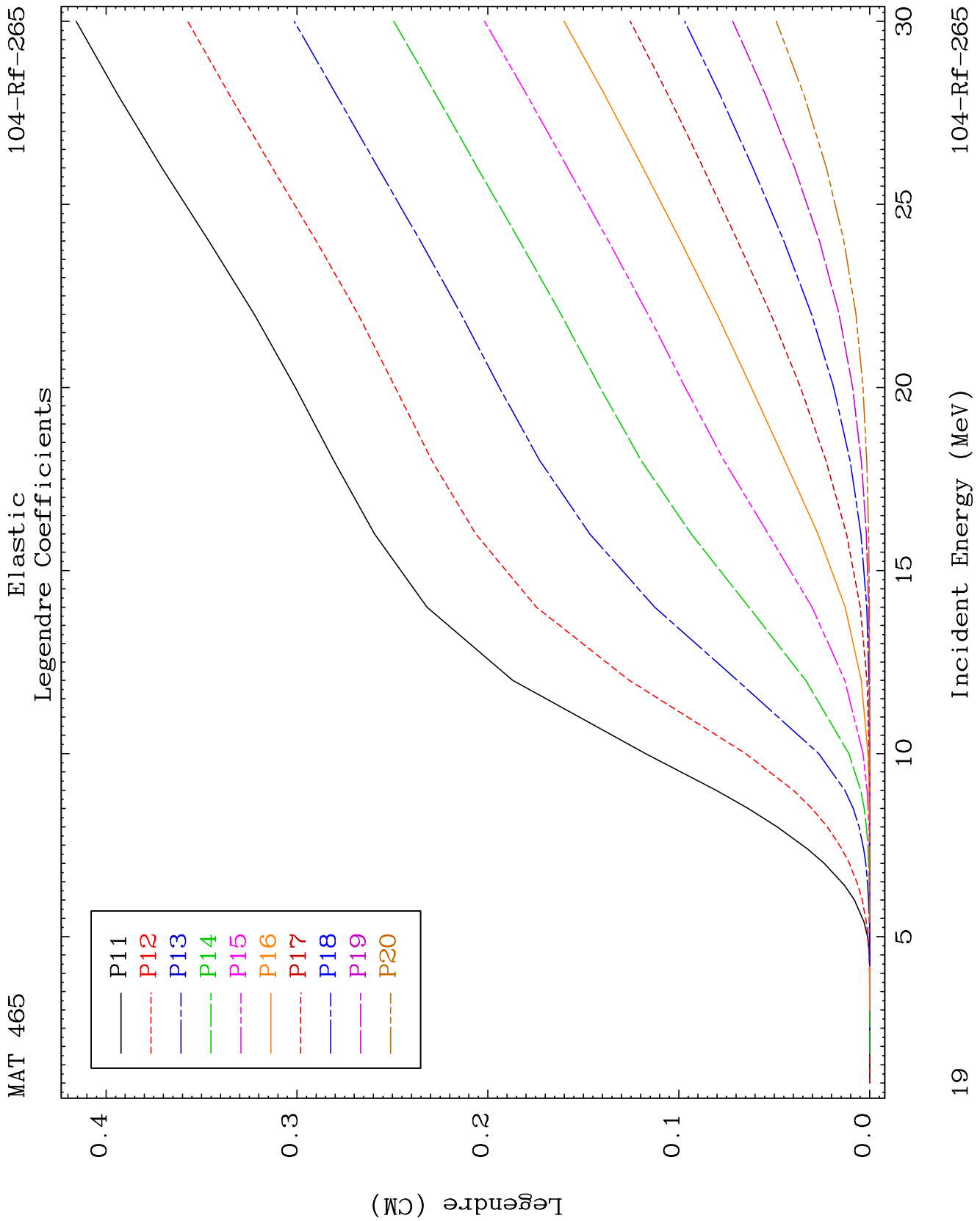


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Incident Energy (MeV)

104-Rf-265





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Elastic Legendre Coefficients

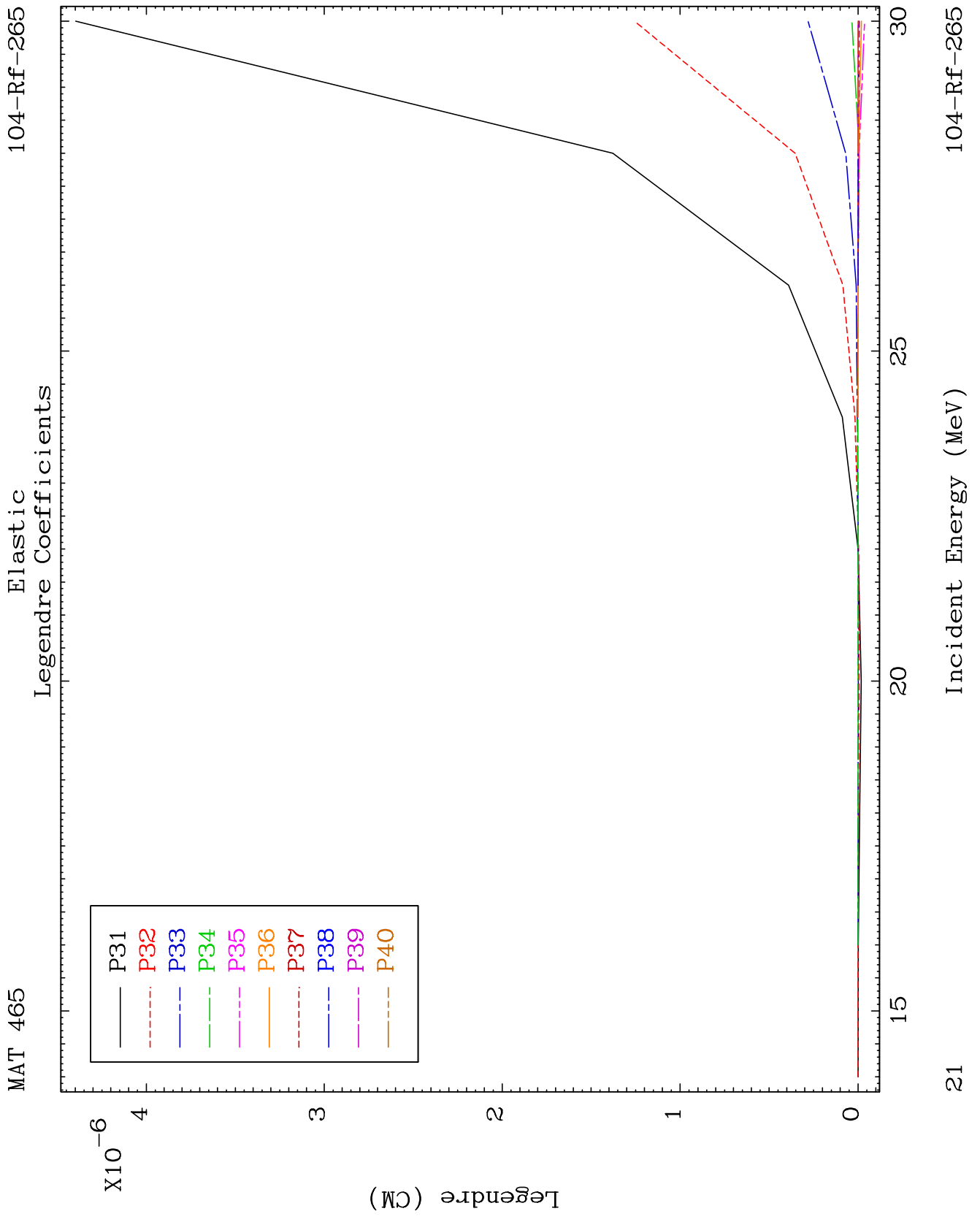
104-Rf-265

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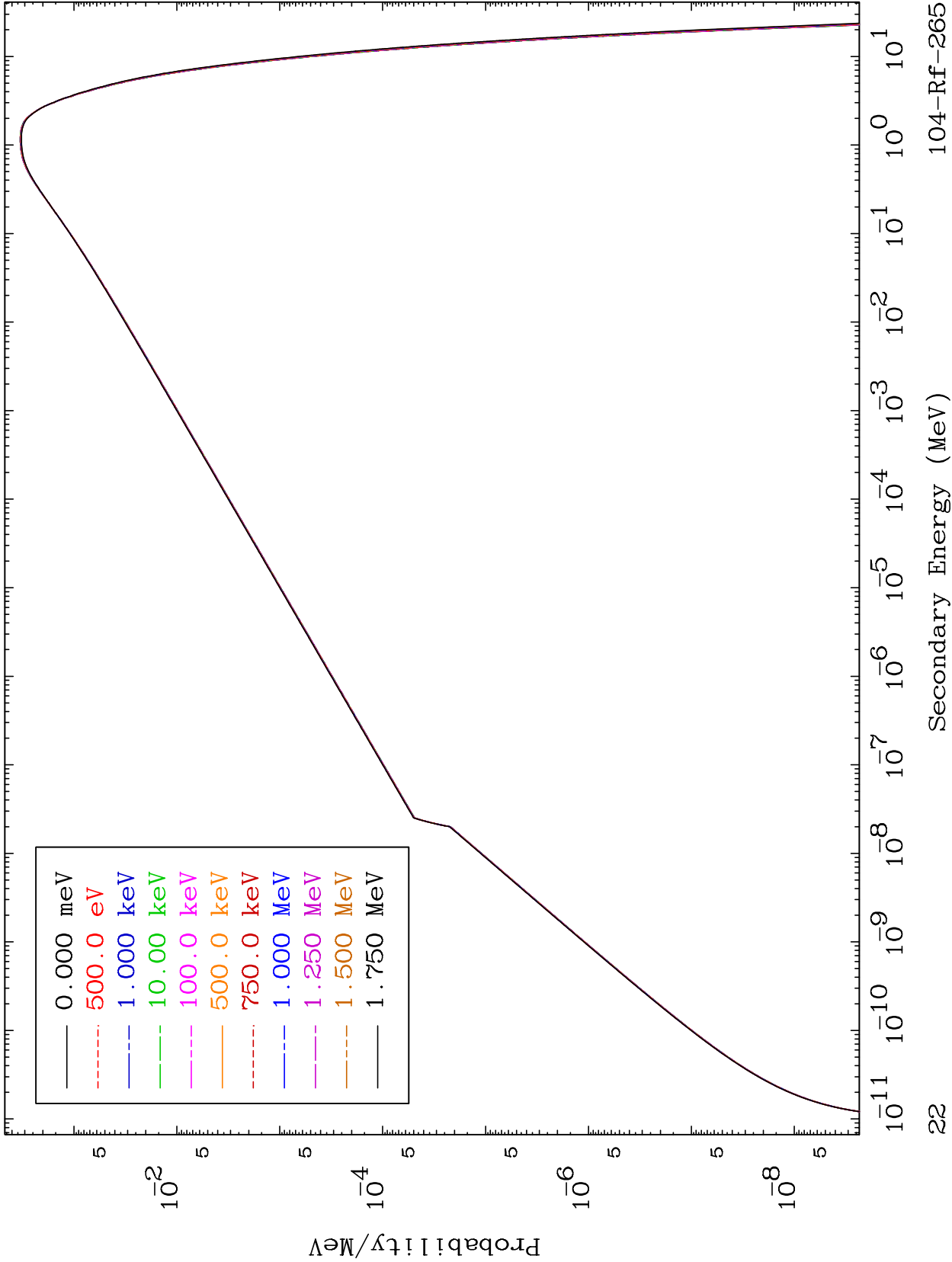
Incident Energy (MeV)

104-Rf-265





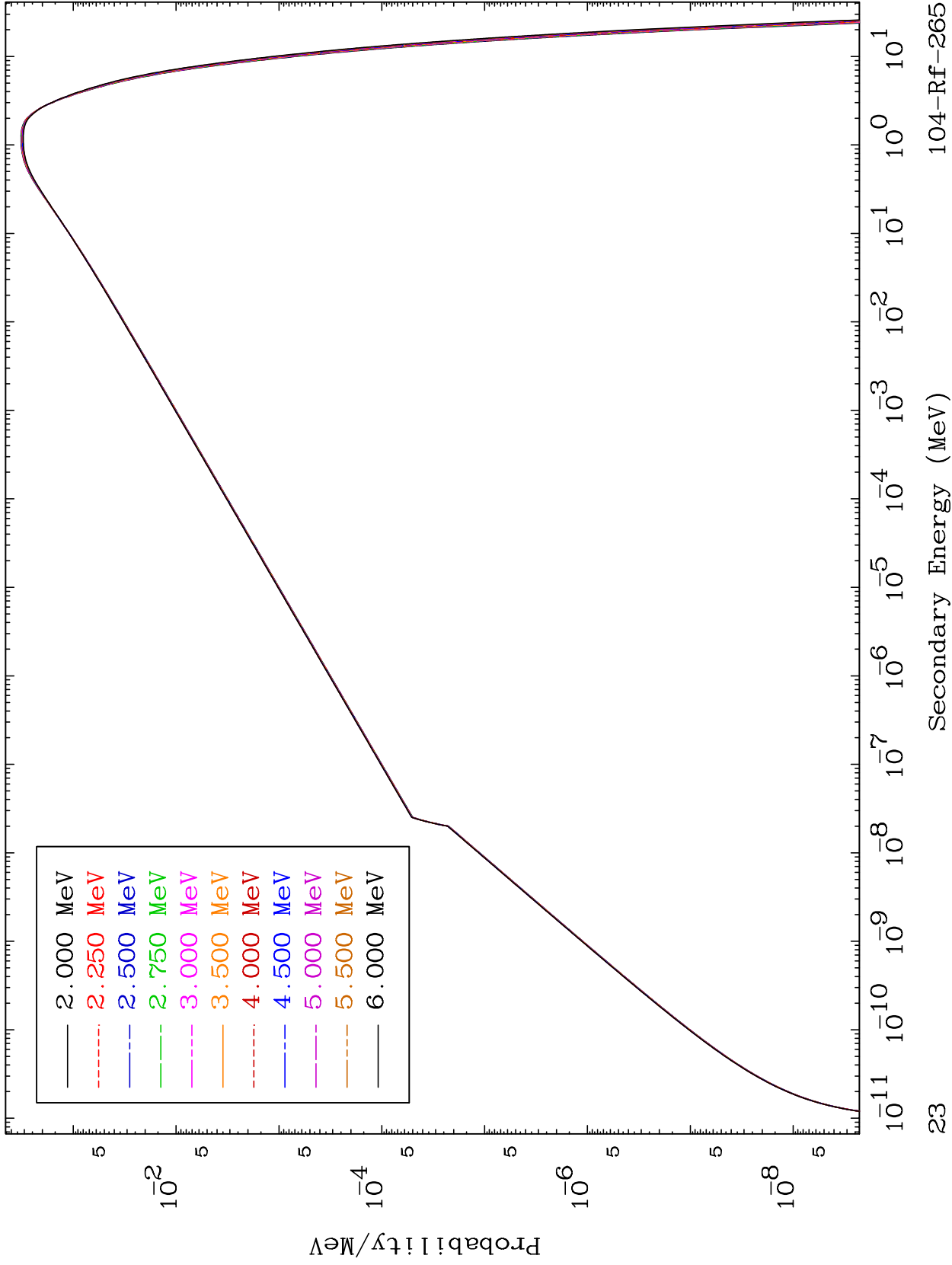
Fission Energy Distributions



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### Fission Energy Distributions

104-Rf-265



104-Rf-265

Secondary Energy (MeV)

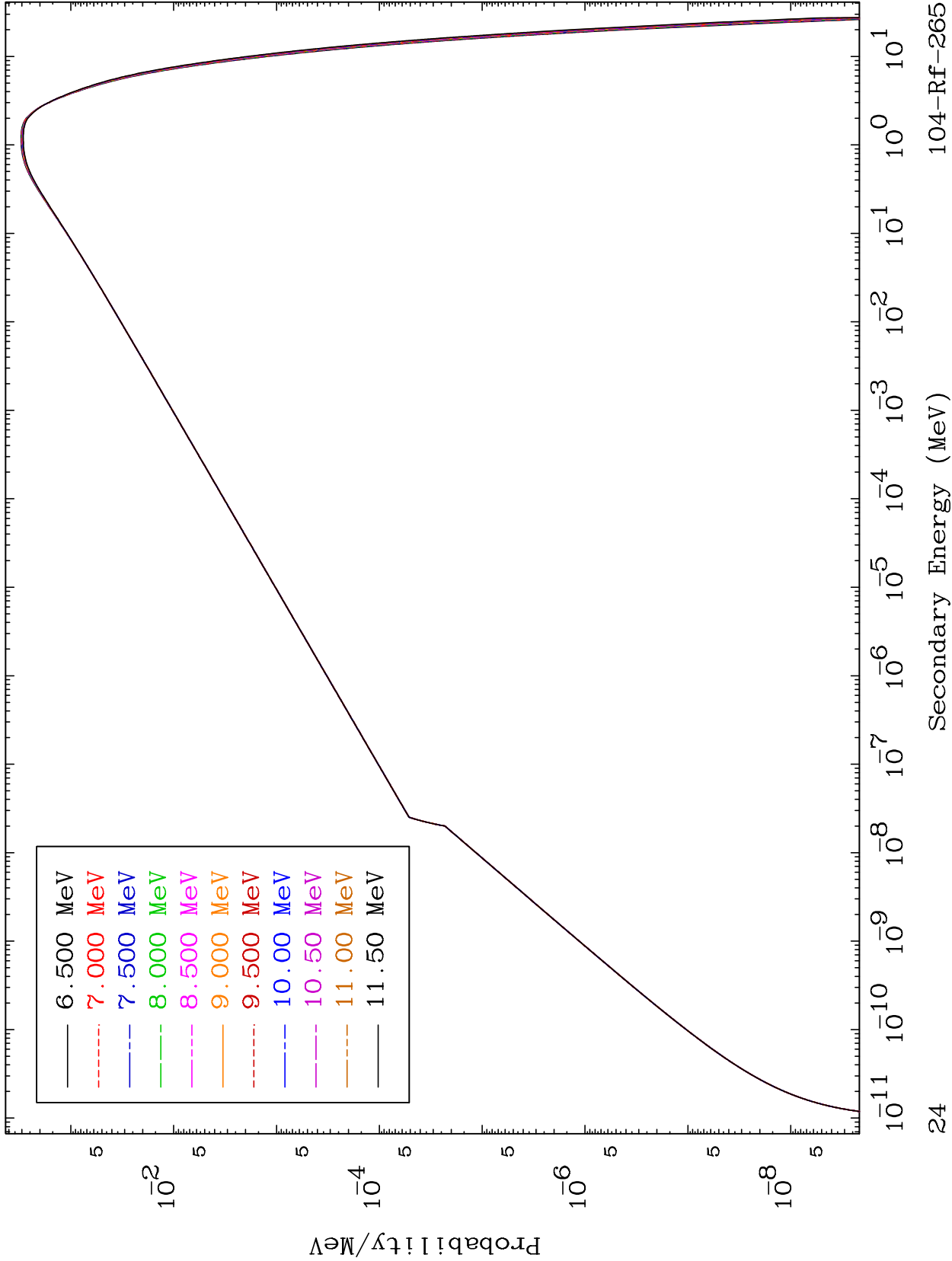
23



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### Fission Energy Distributions

104-Rf-265



104-Rf-265

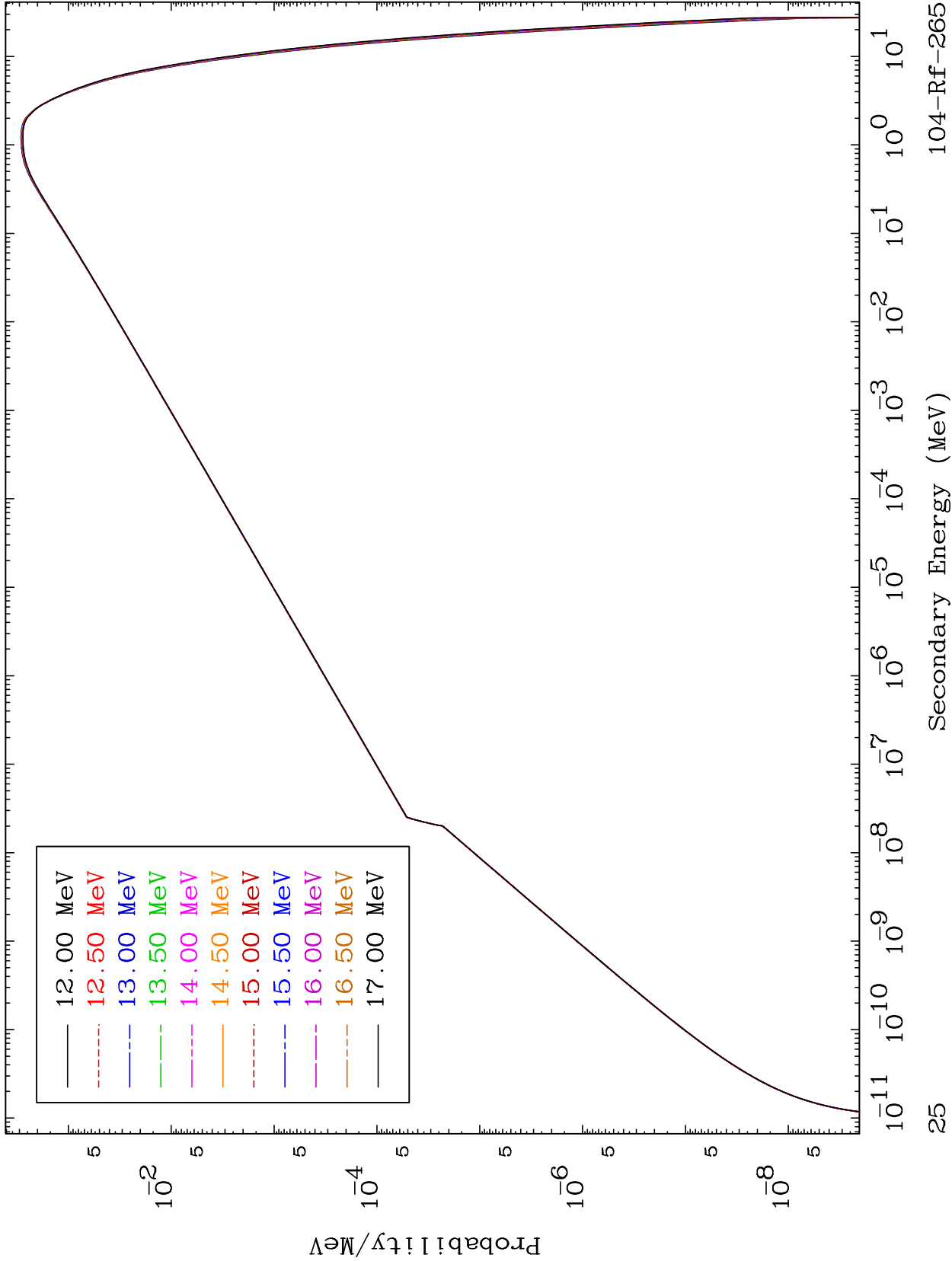
Secondary Energy (MeV)

24

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### Fission Energy Distributions

104-Rf-265



104-Rf-265

Secondary Energy (MeV)

25

Fission Energy Distributions

