

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

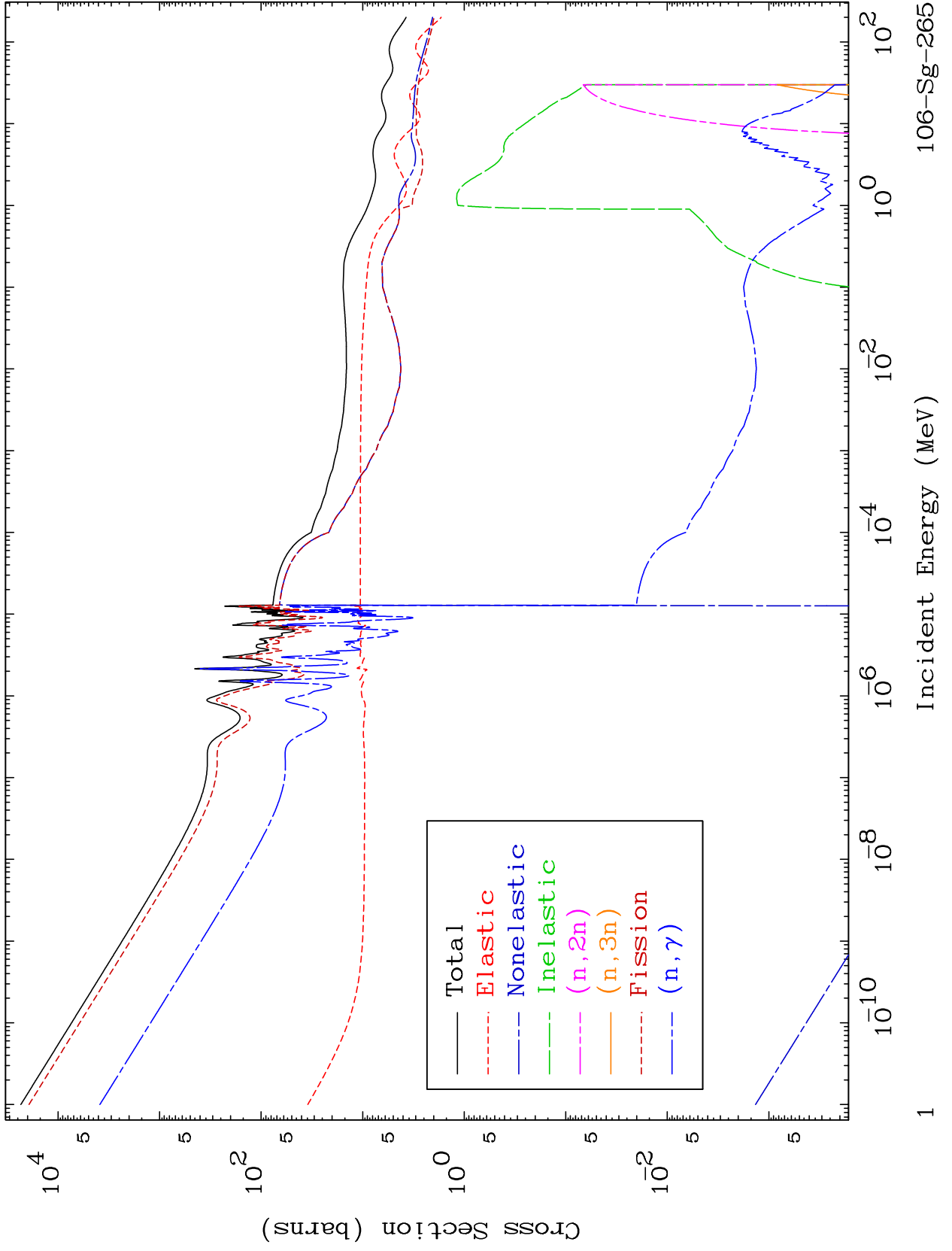
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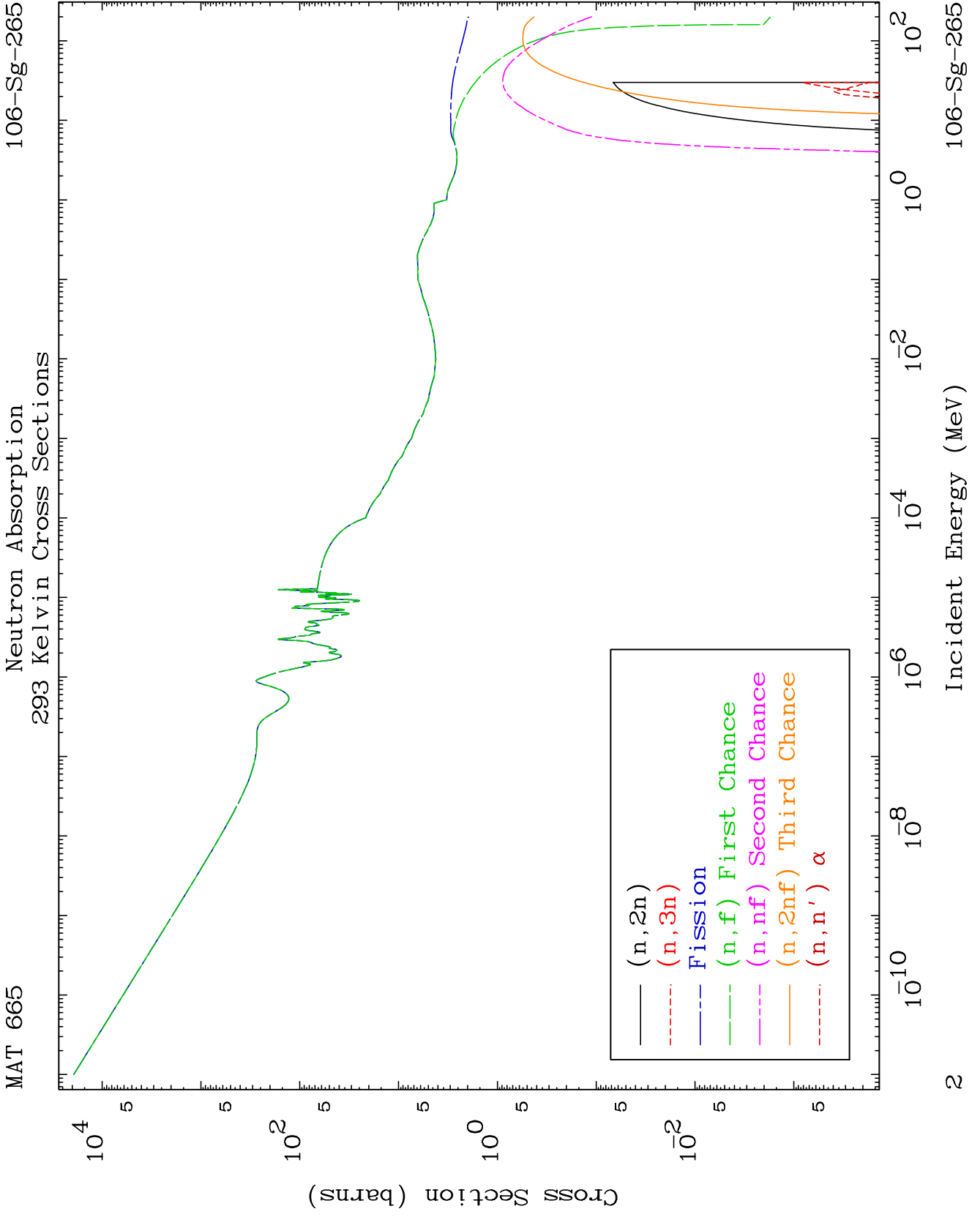
Press Mouse Button to Start

MAT 665

Neutron Major  
293 Kelvin Cross Sections

106-Sg-265

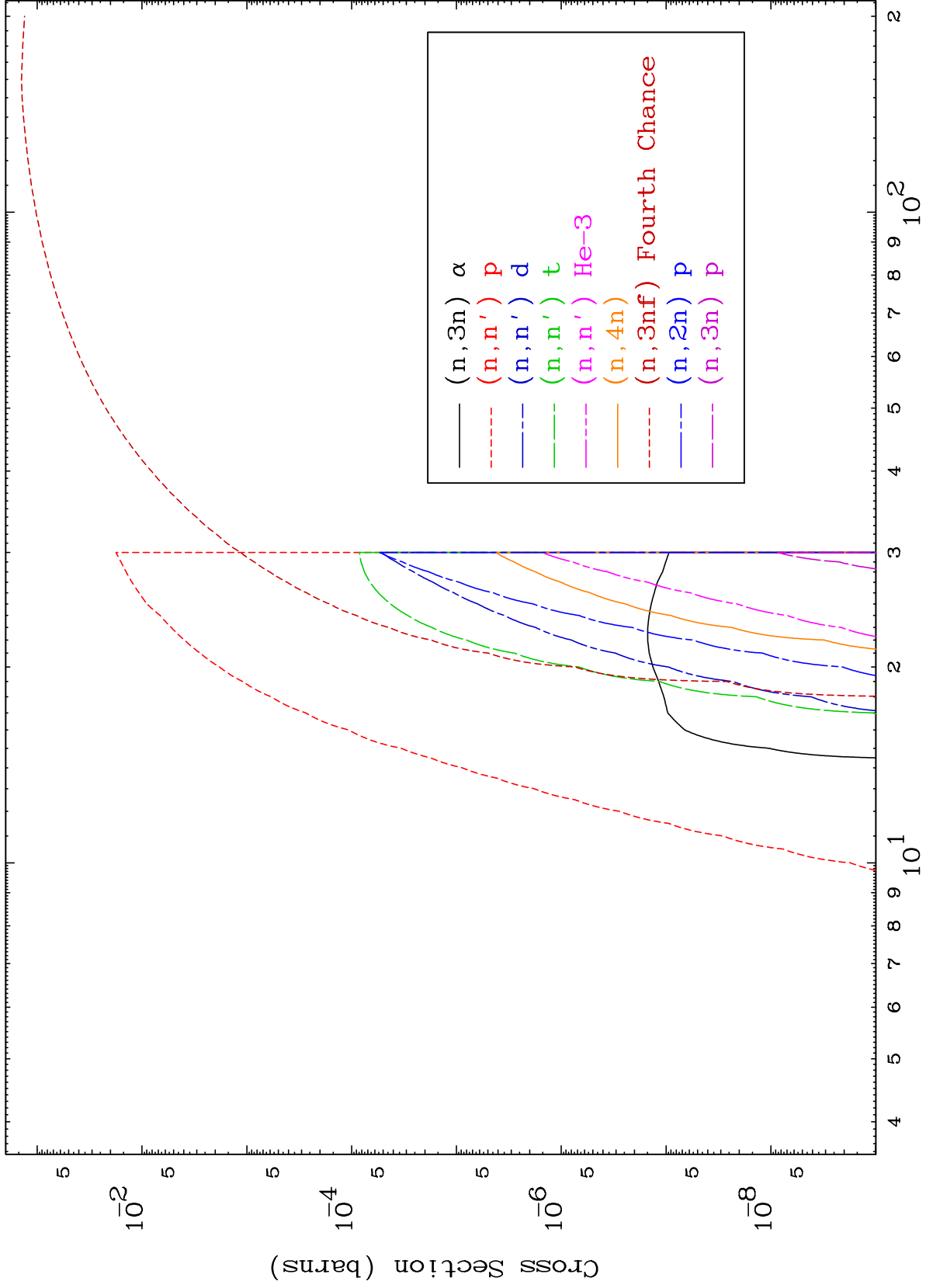


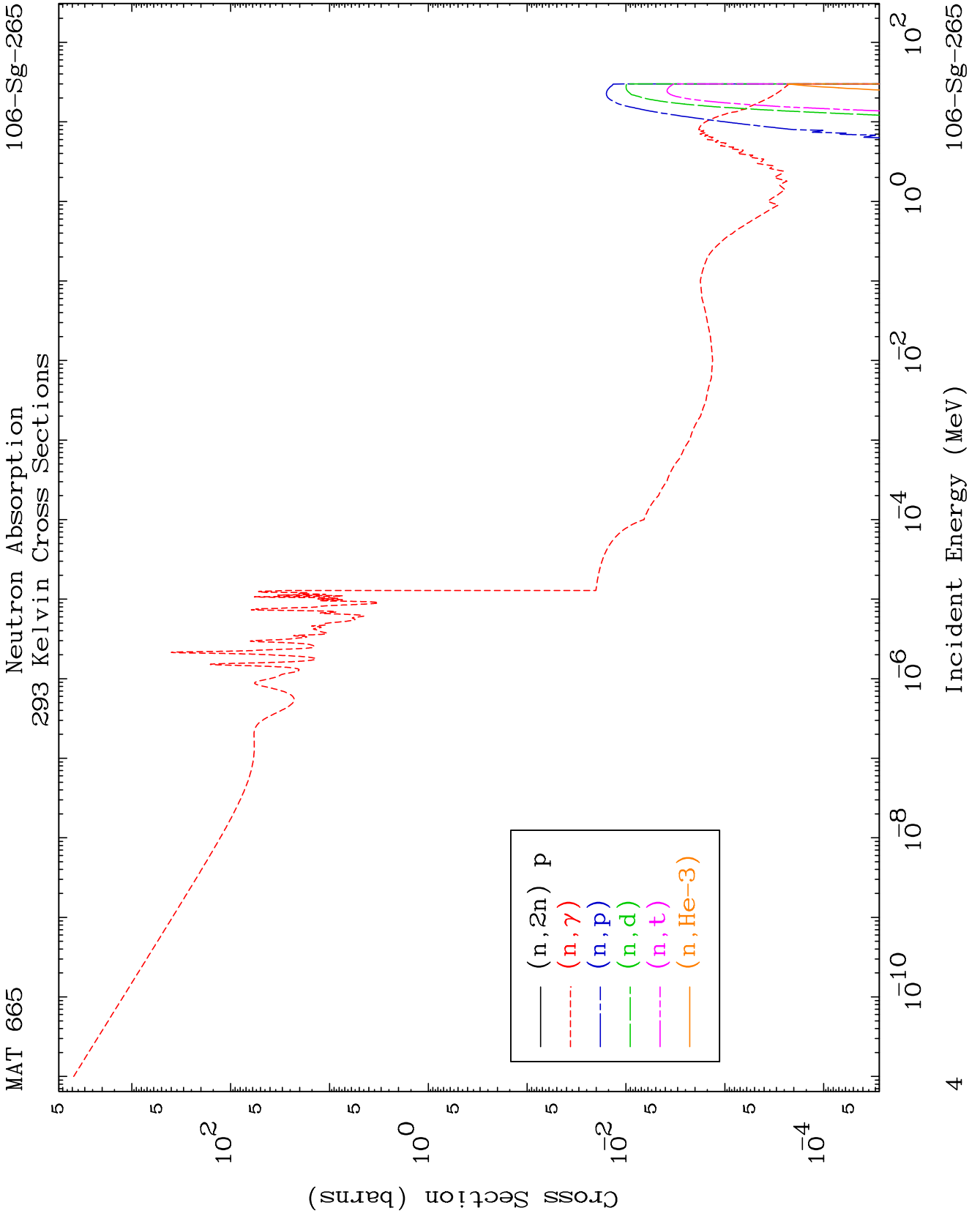


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Neutron Absorption  
293 Kelvin Cross Sections

106-Sg-265

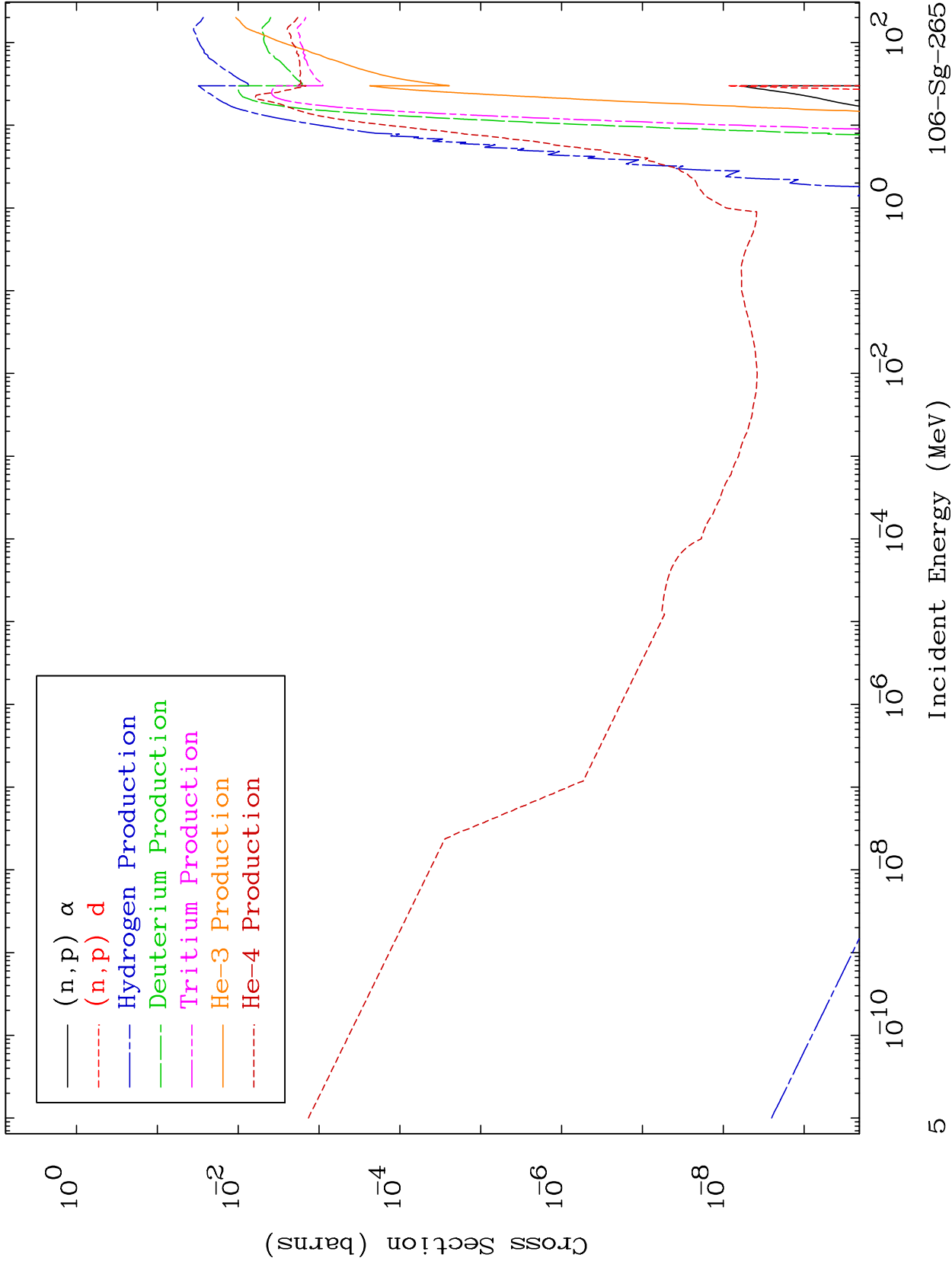


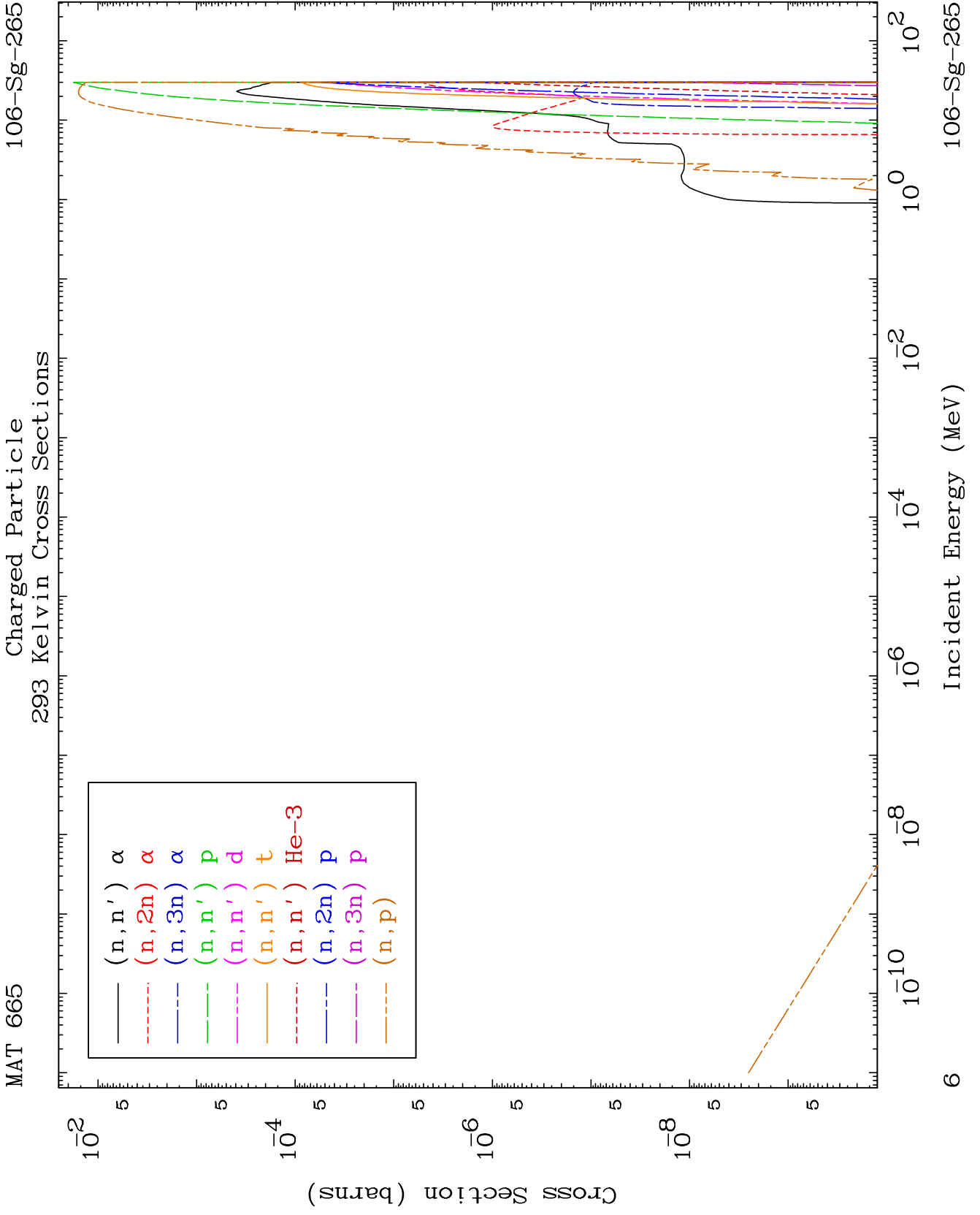


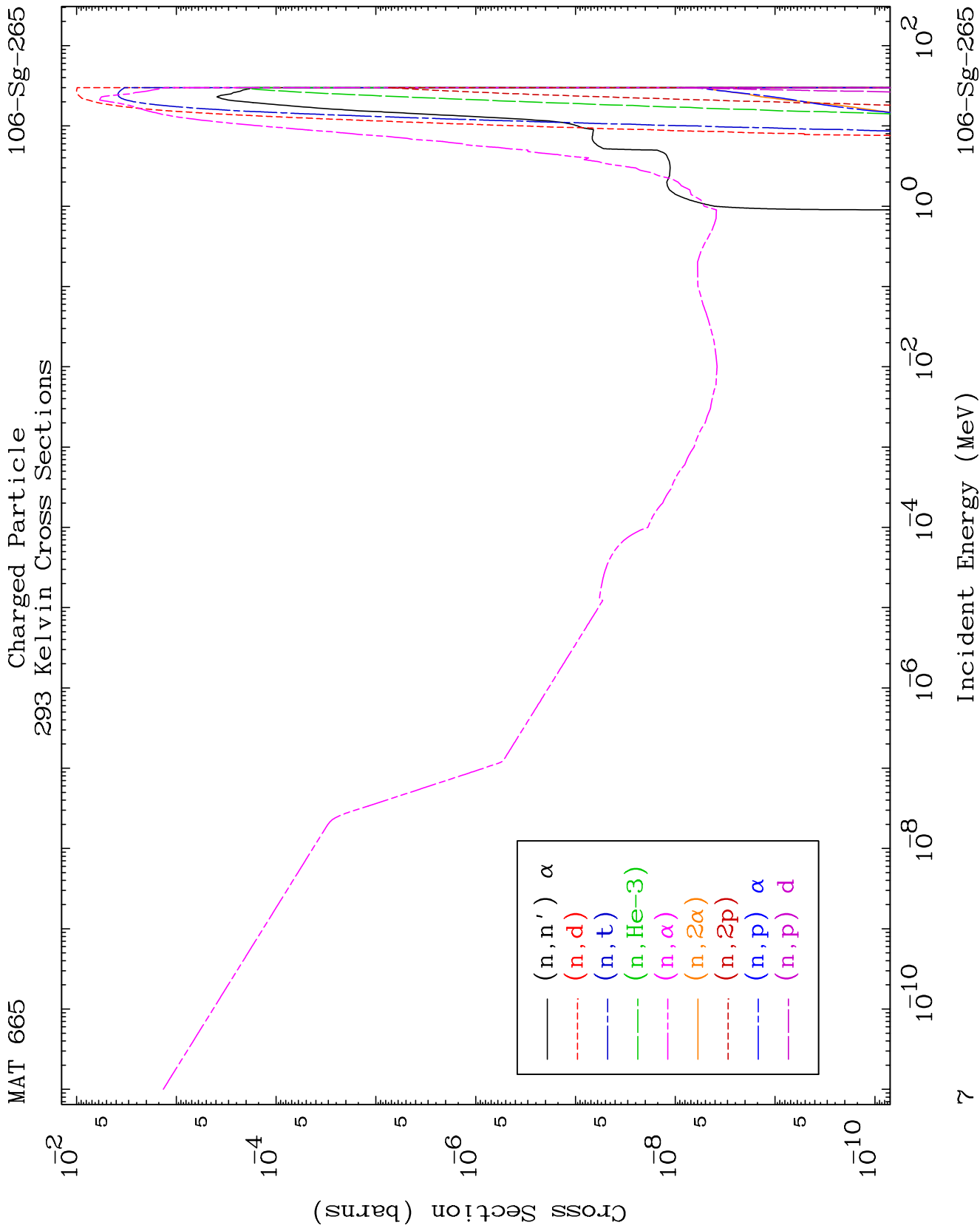
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Neutron Absorption  
293 Kelvin Cross Sections

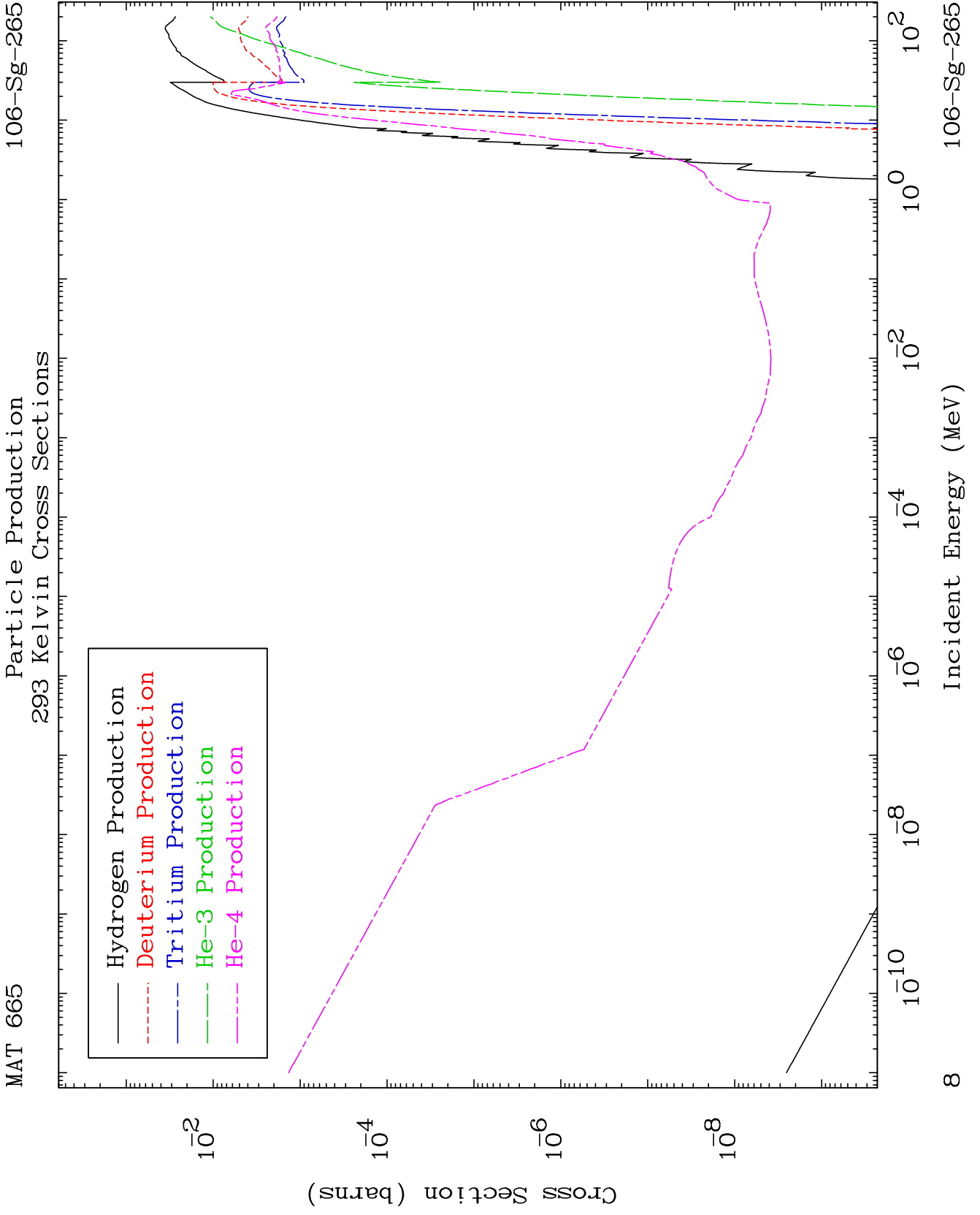
106-Sg-265

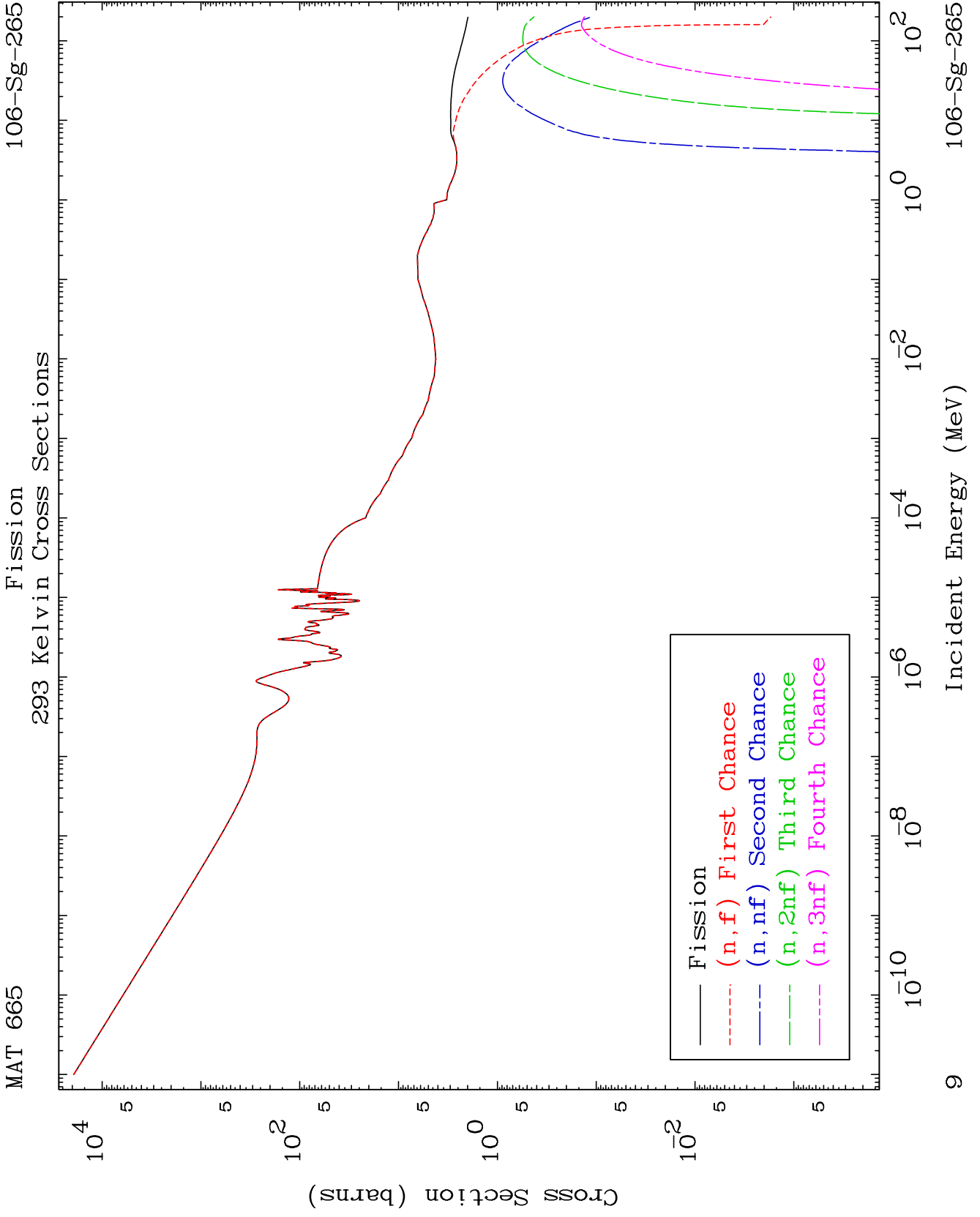


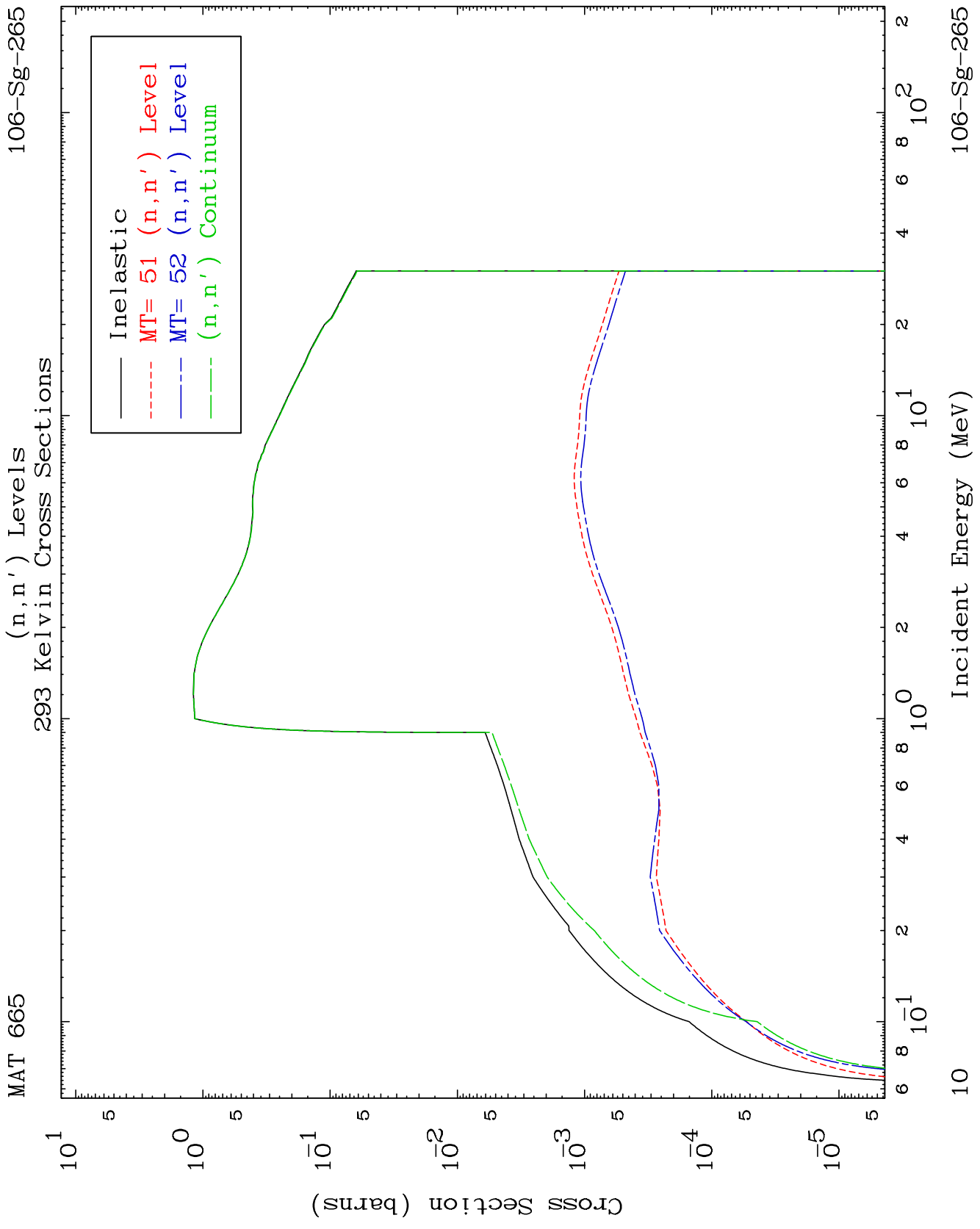


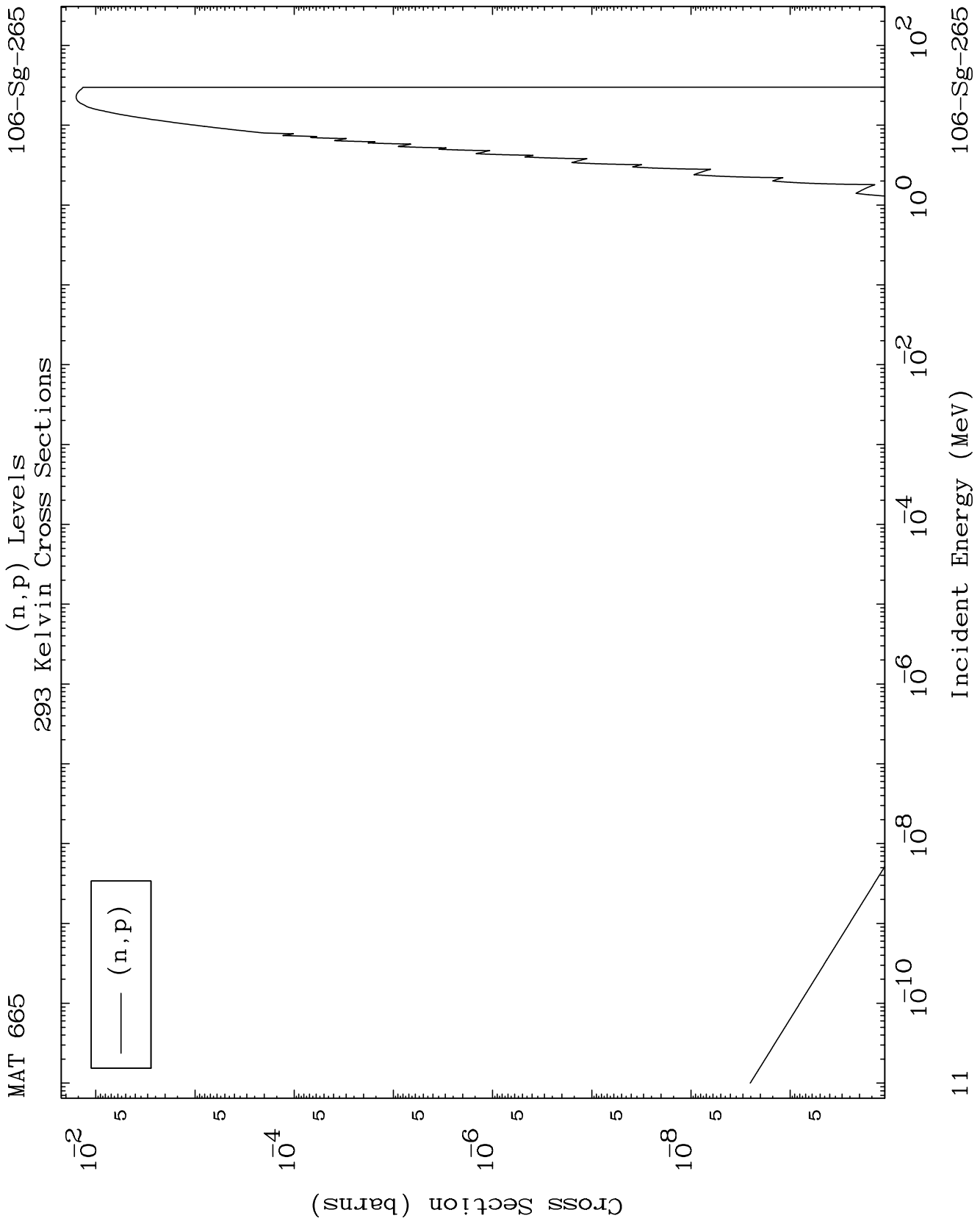


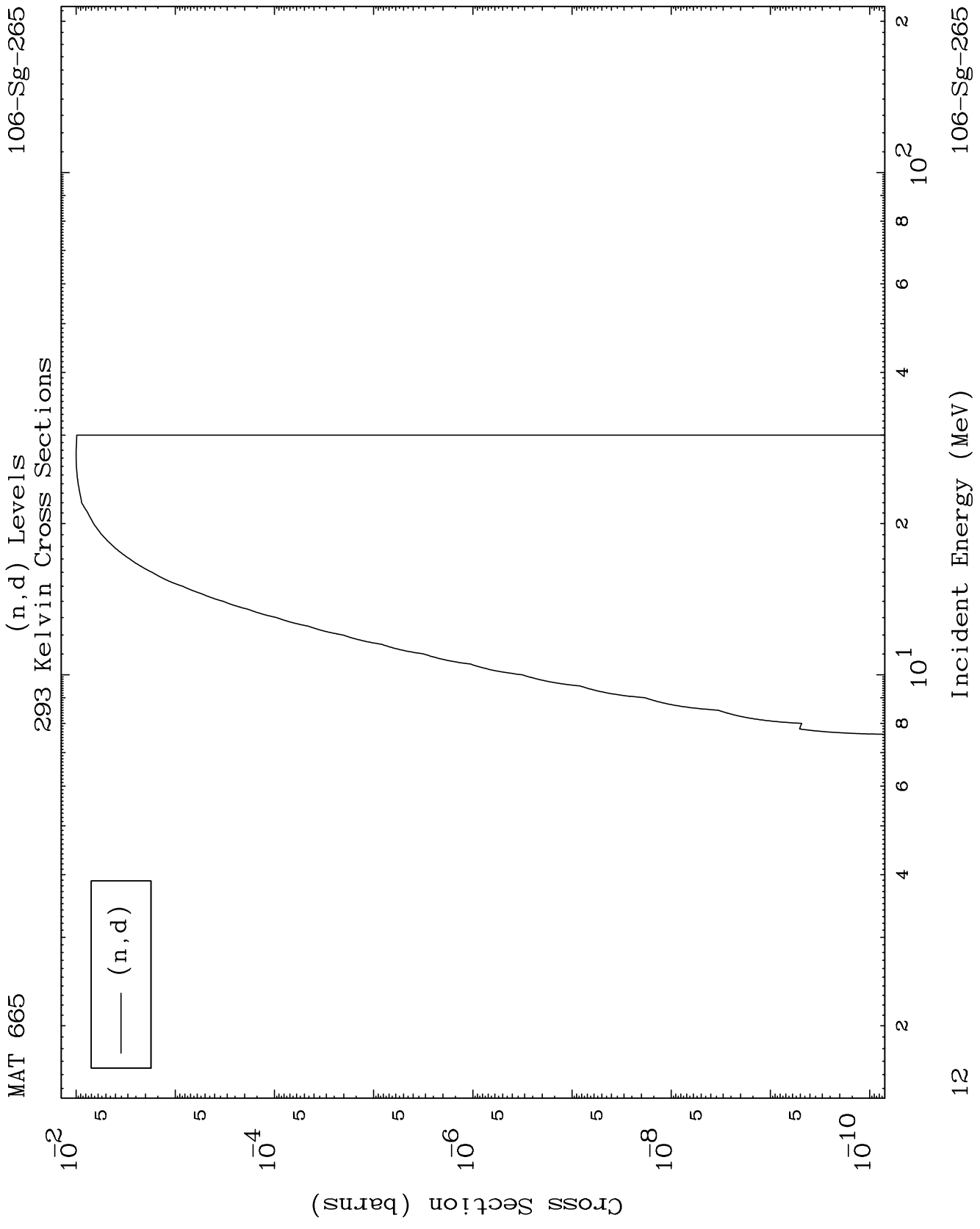








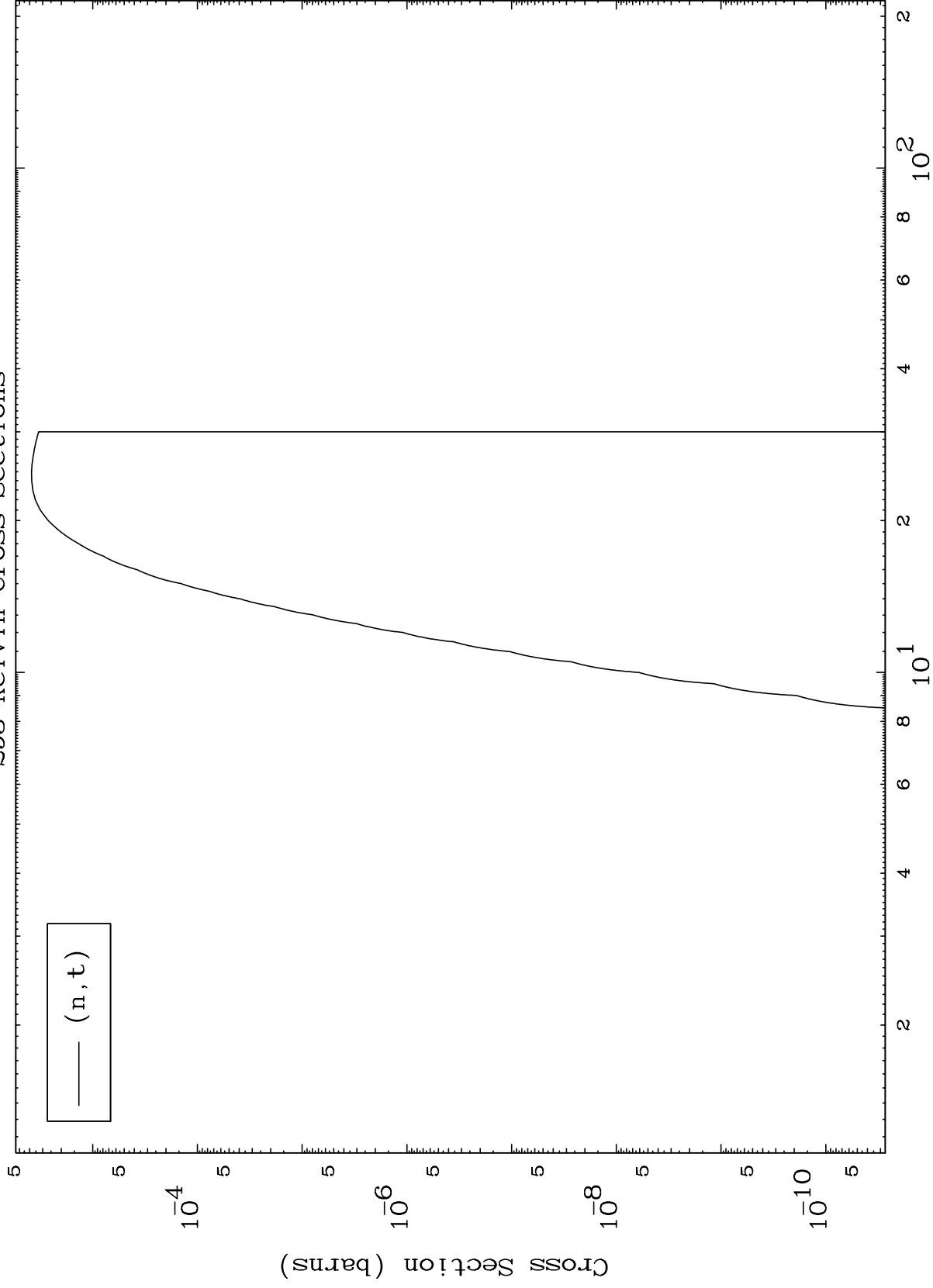




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(n,t) Levels  
293 Kelvin Cross Sections

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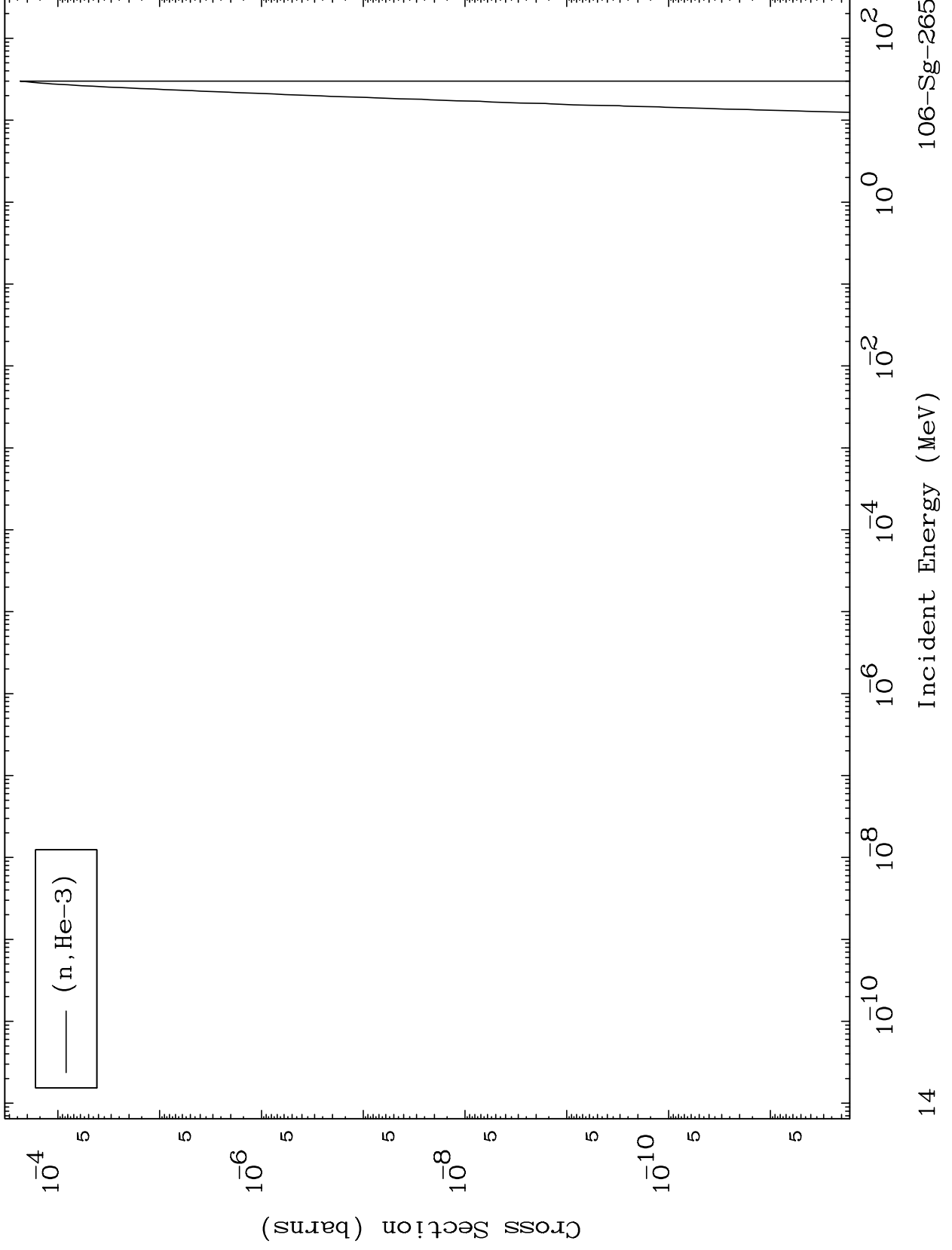
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

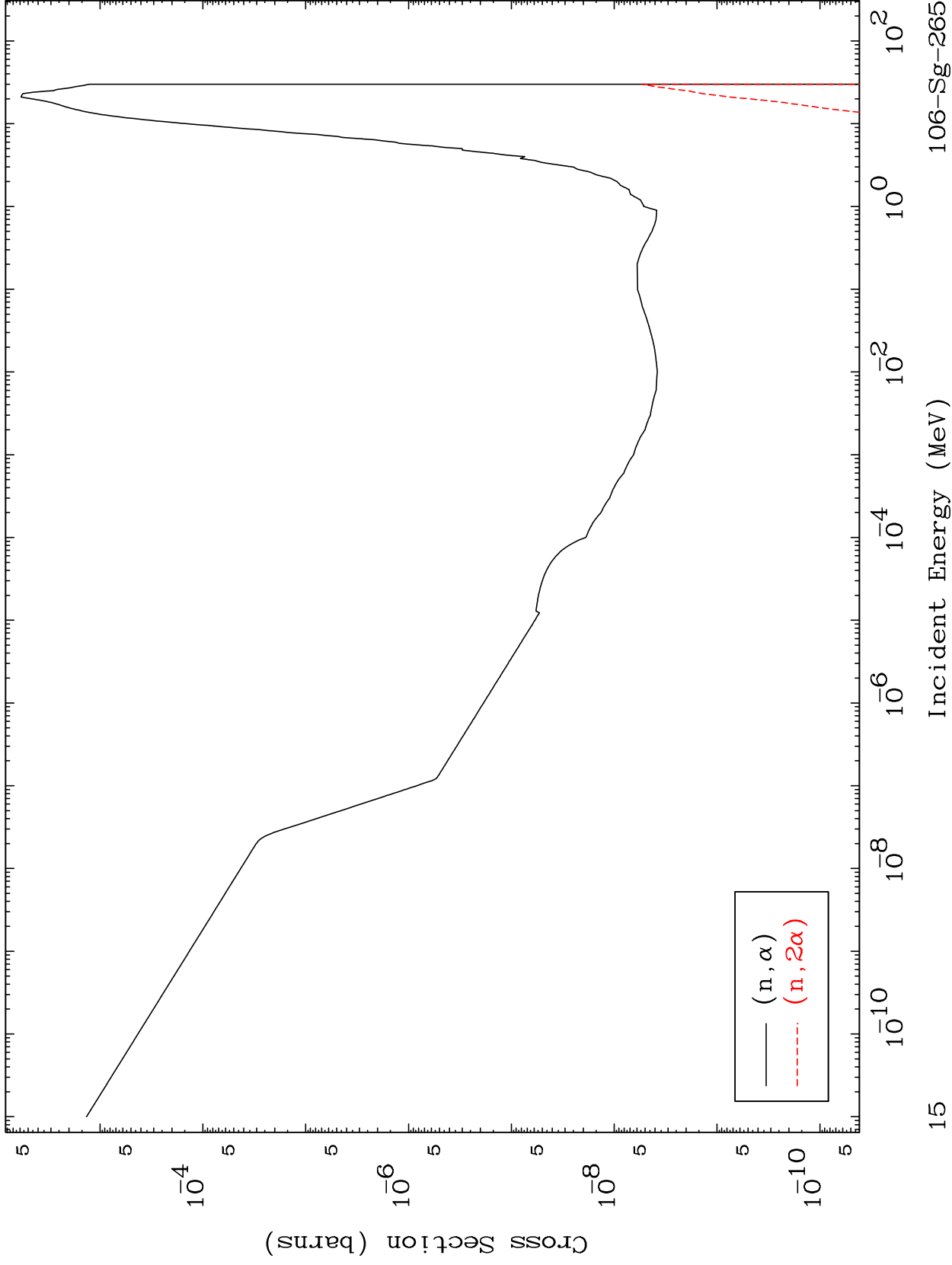
106-Sg-265



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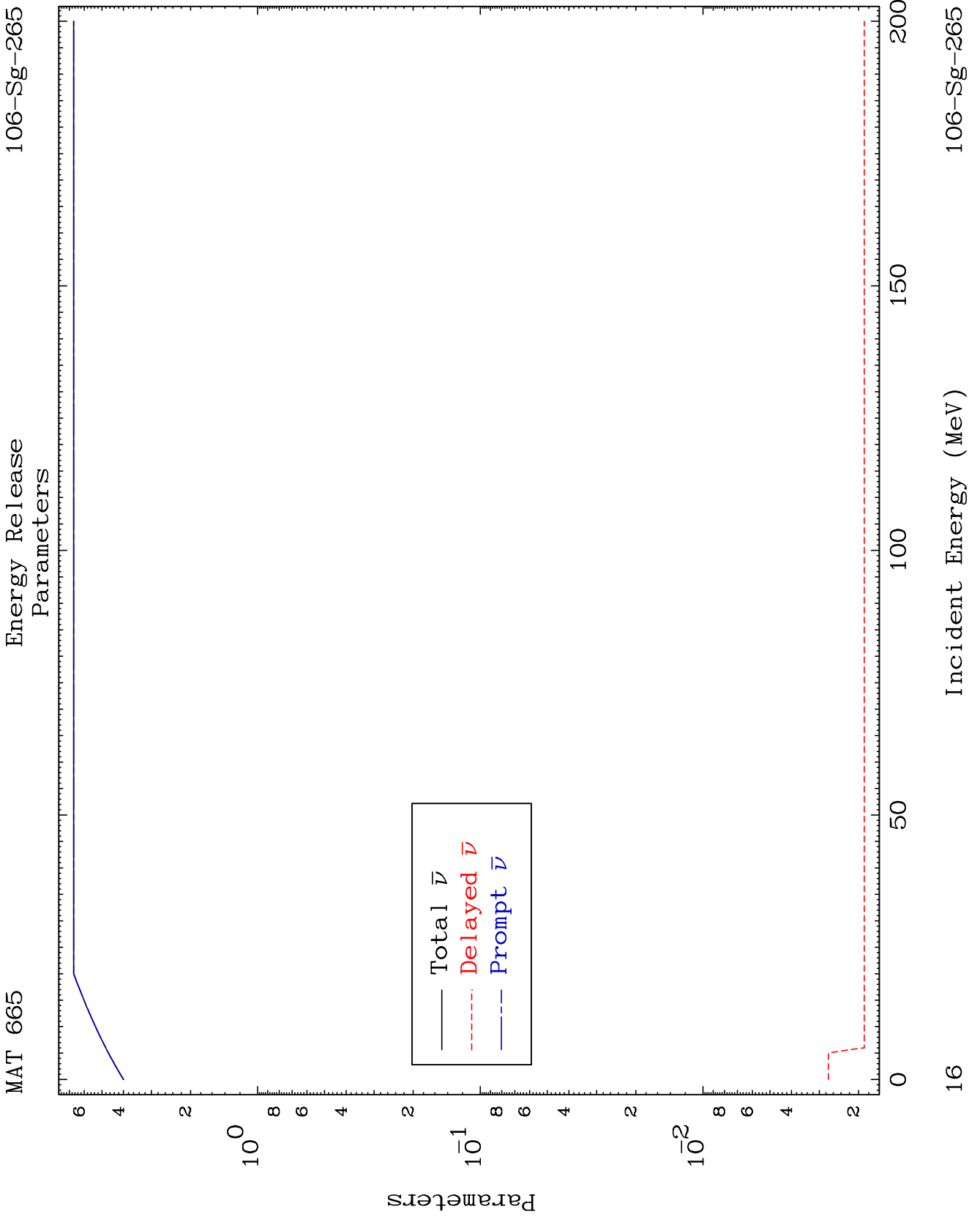
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

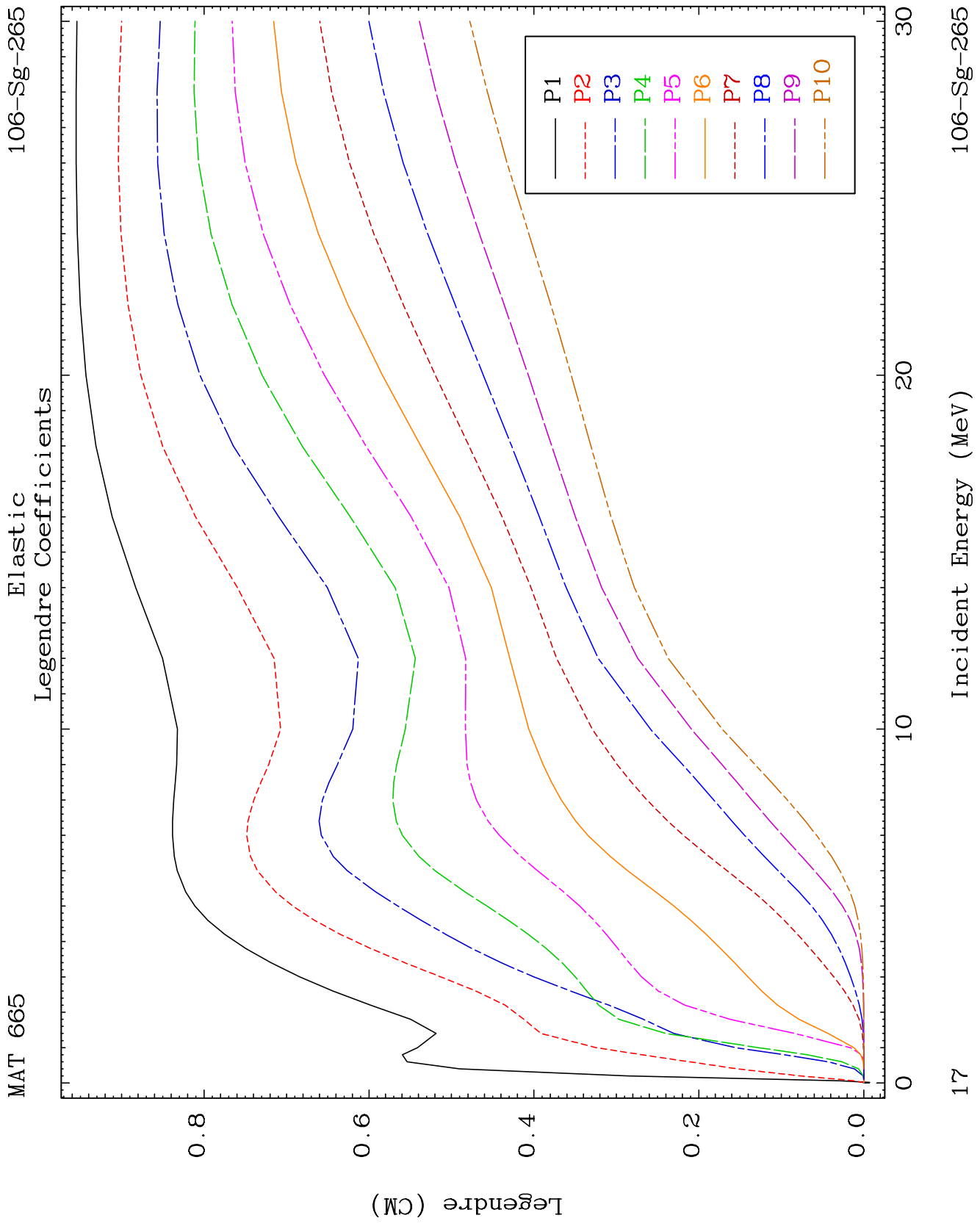
106-Sg-265

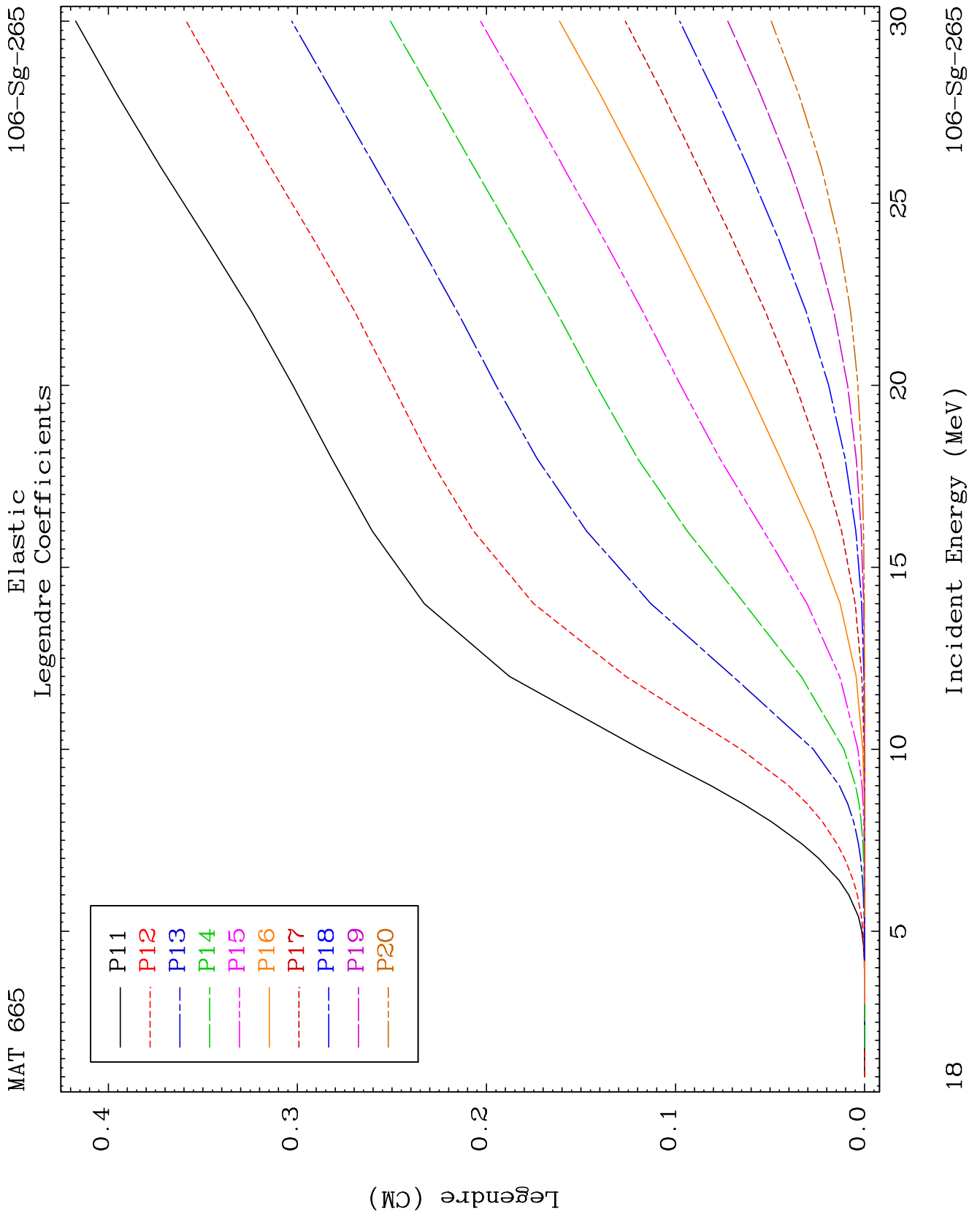


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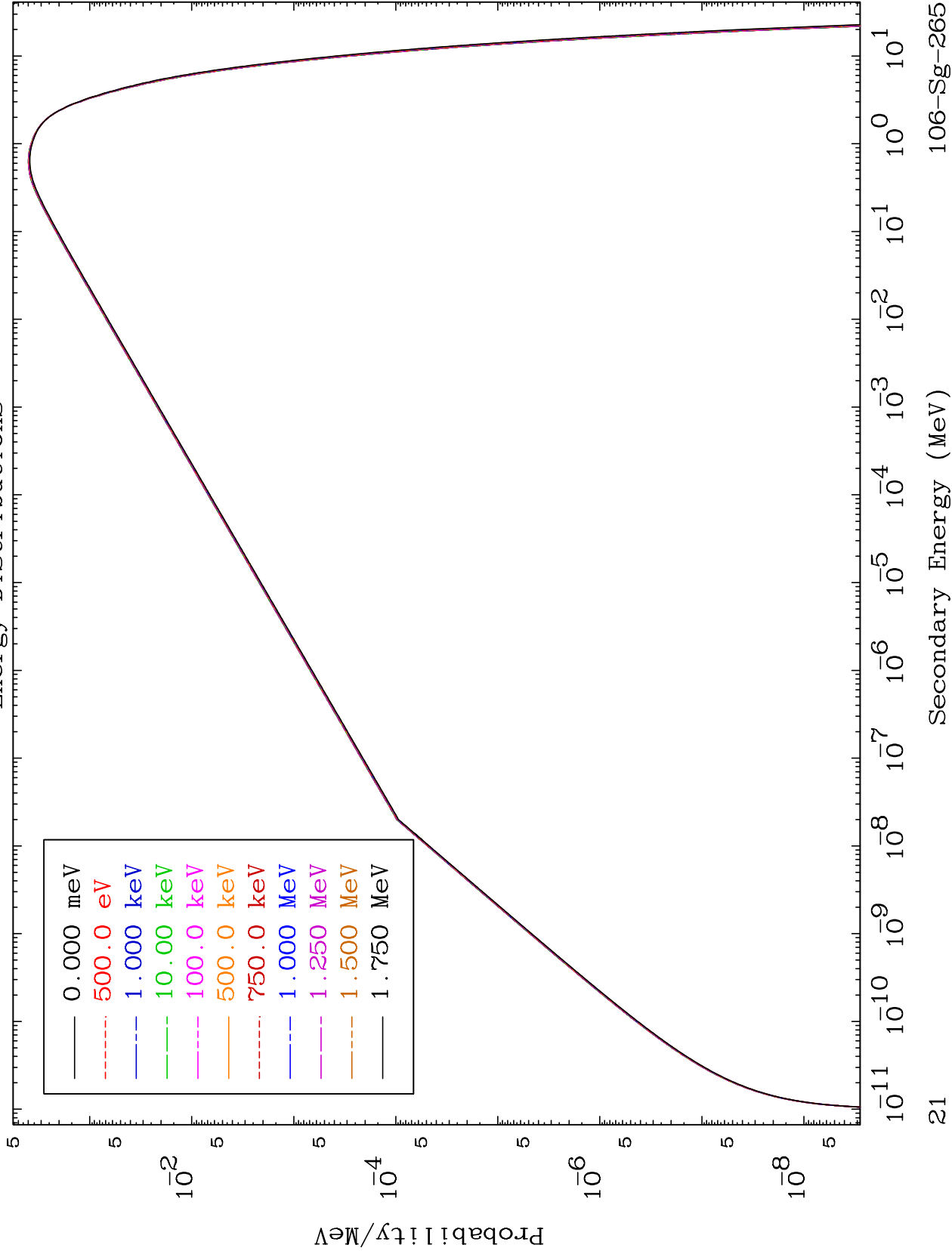








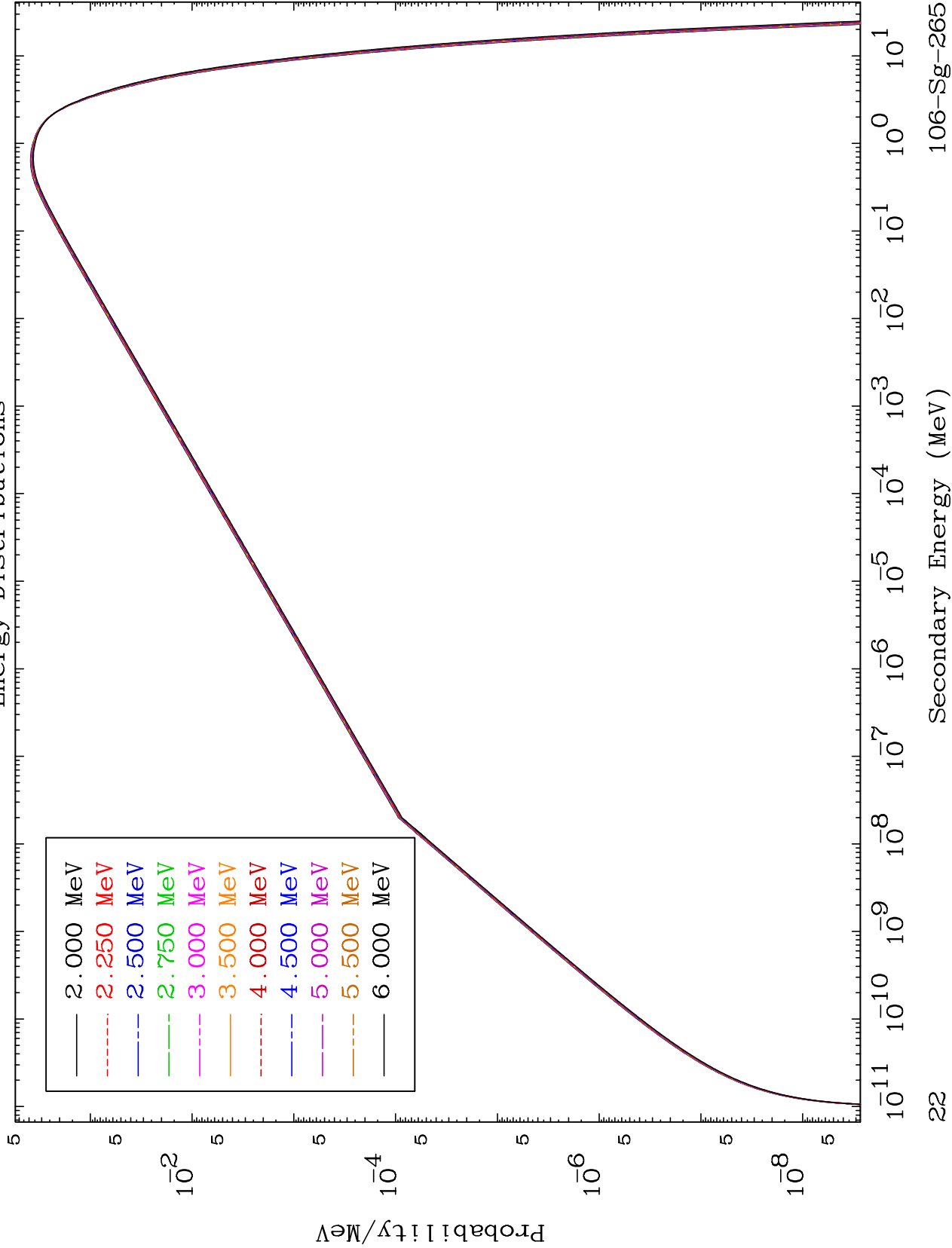
Fission Energy Distributions



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### Fission Energy Distributions

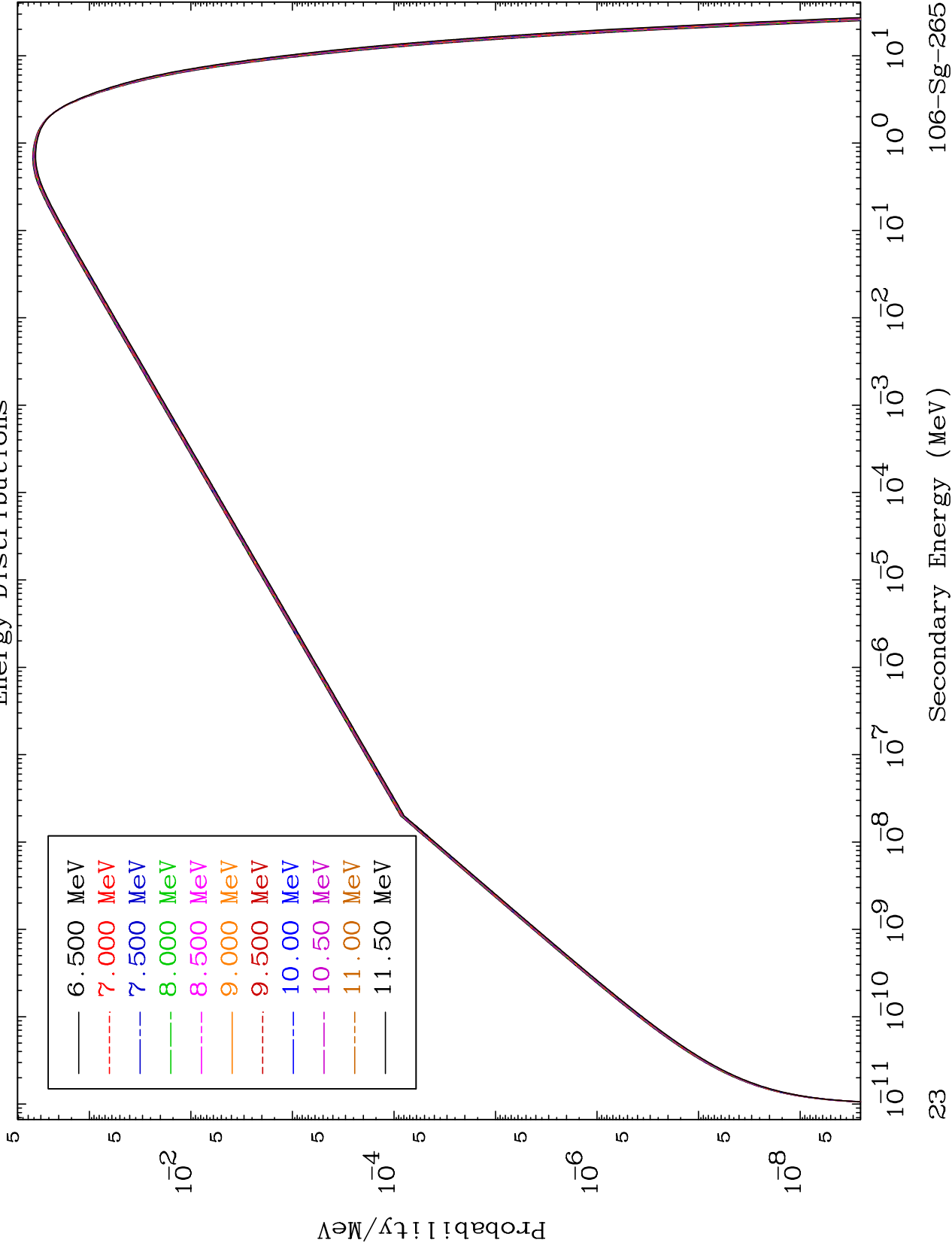
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### Fission Energy Distributions

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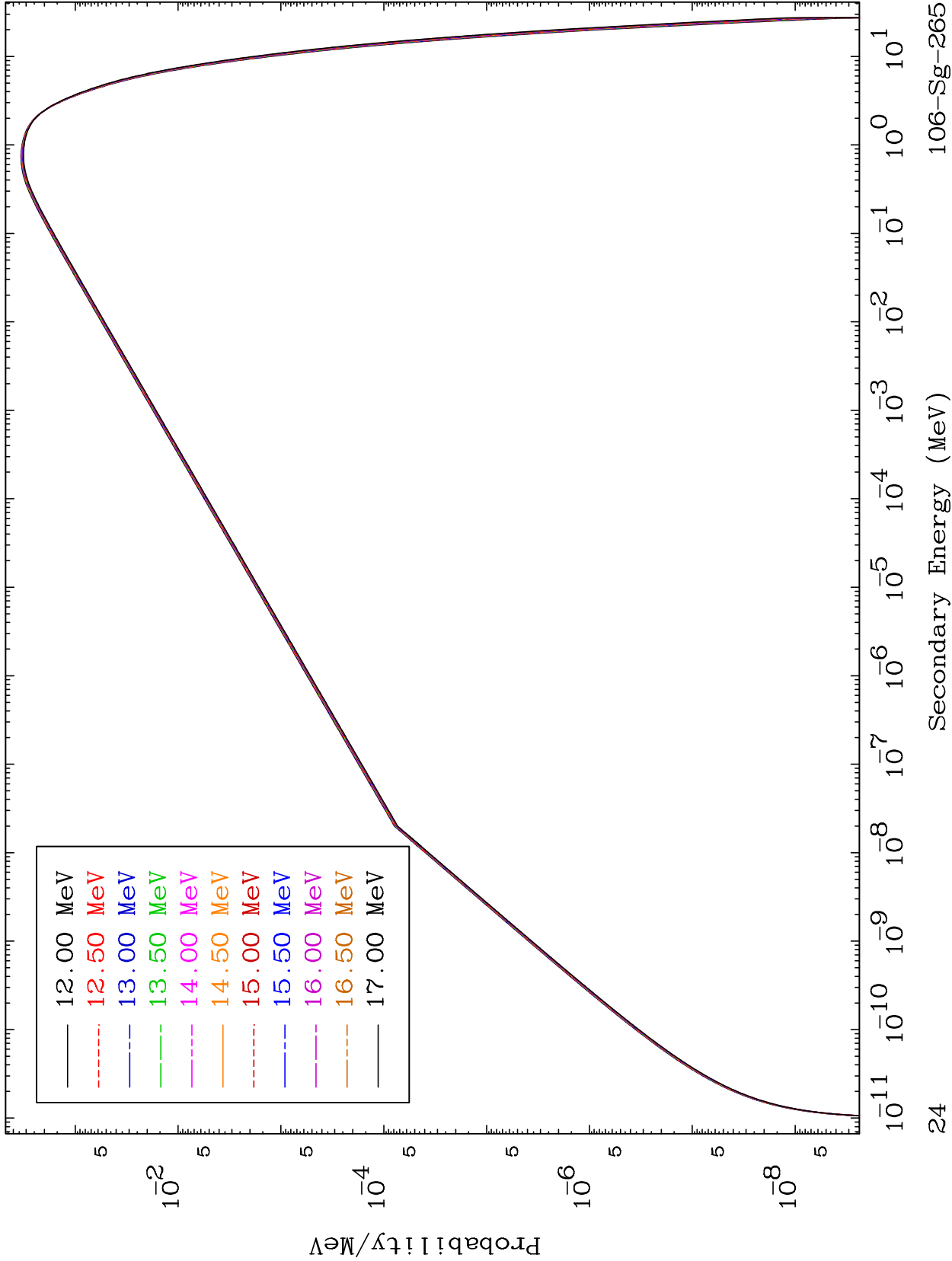
106-Sg-265



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### Fission Energy Distributions

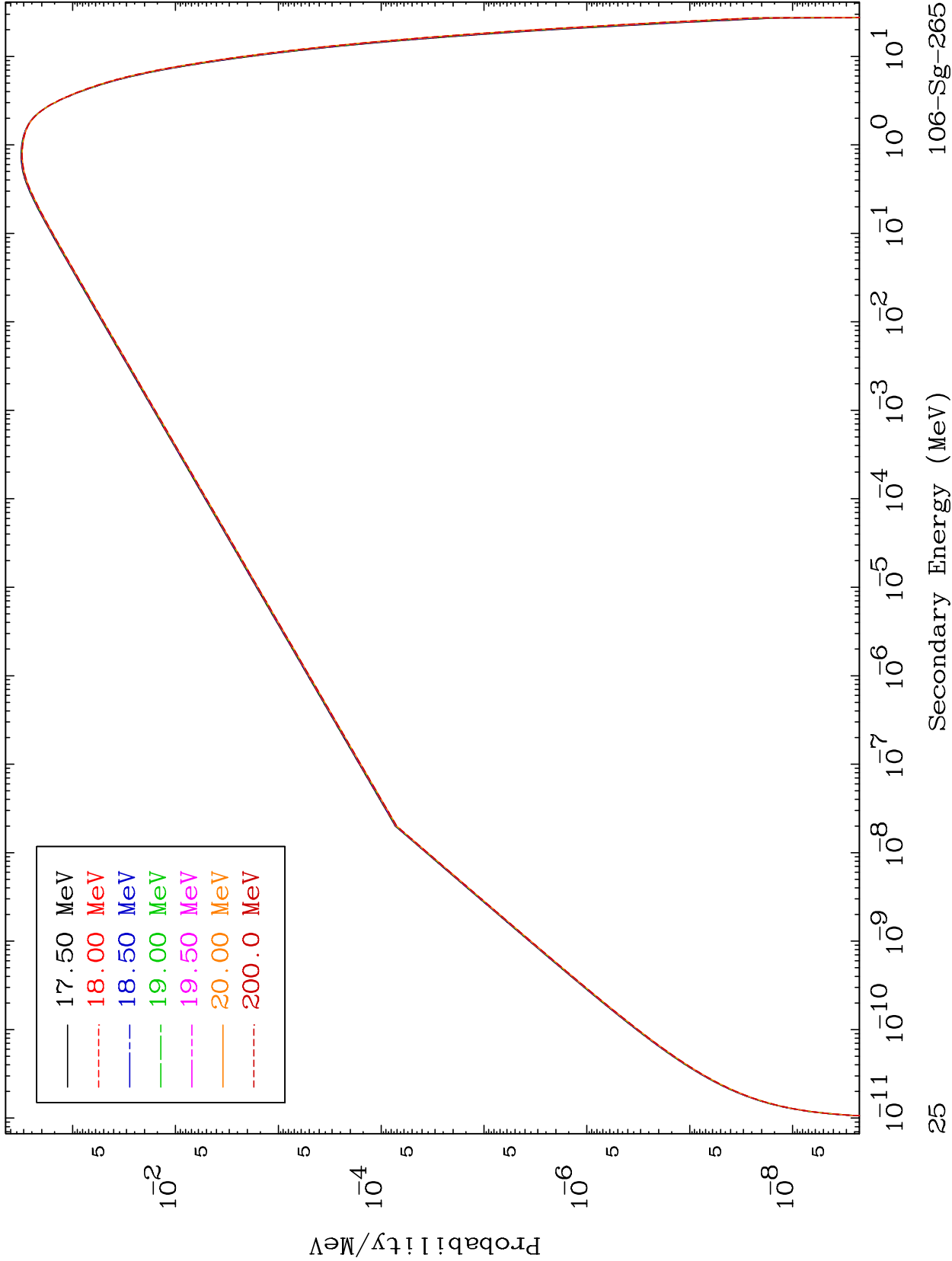
106-Sg-265



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### Fission Energy Distributions

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106-Sg-265

