

Program EVALPLOT
(Version 2021-1)

by

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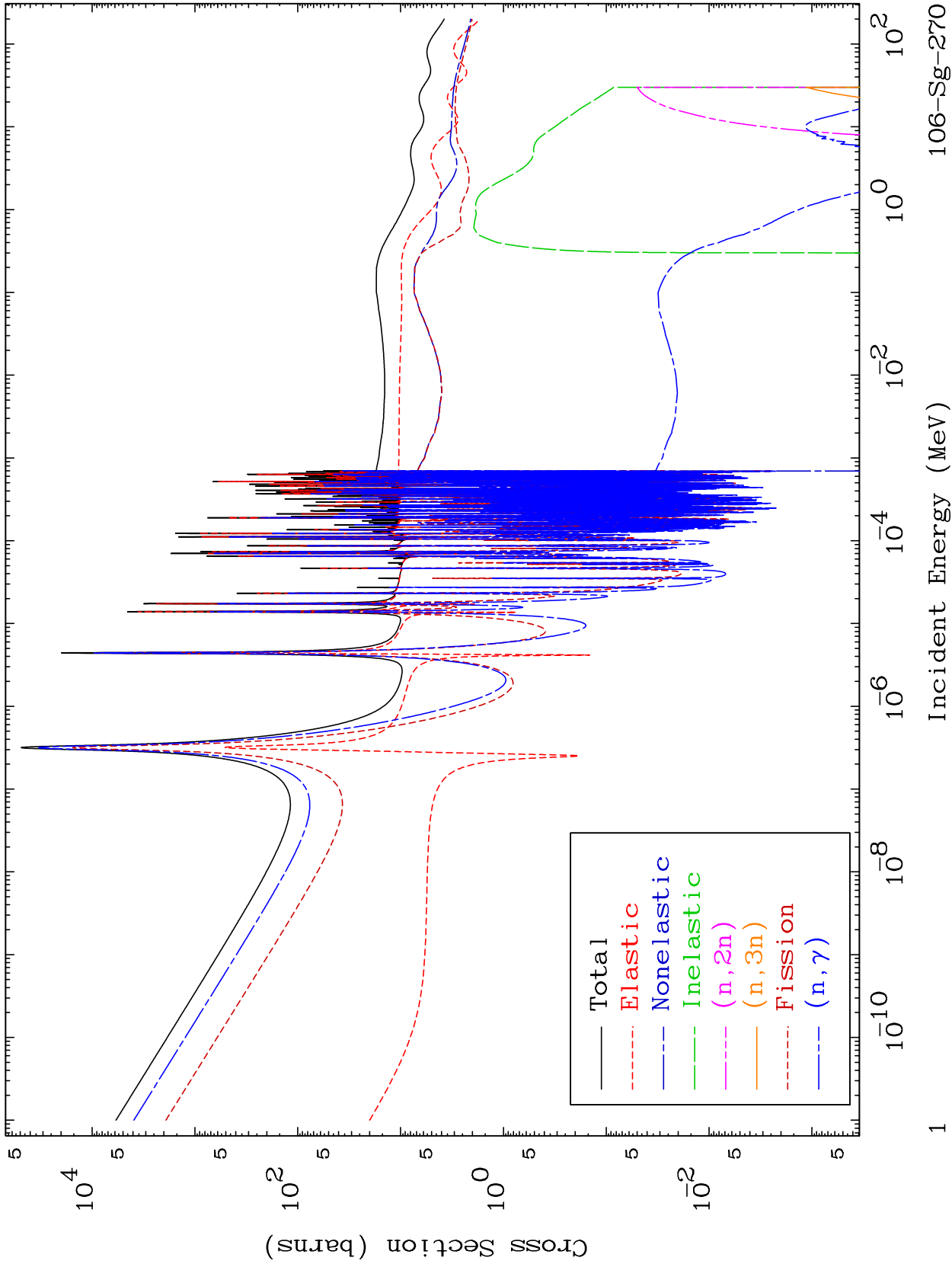
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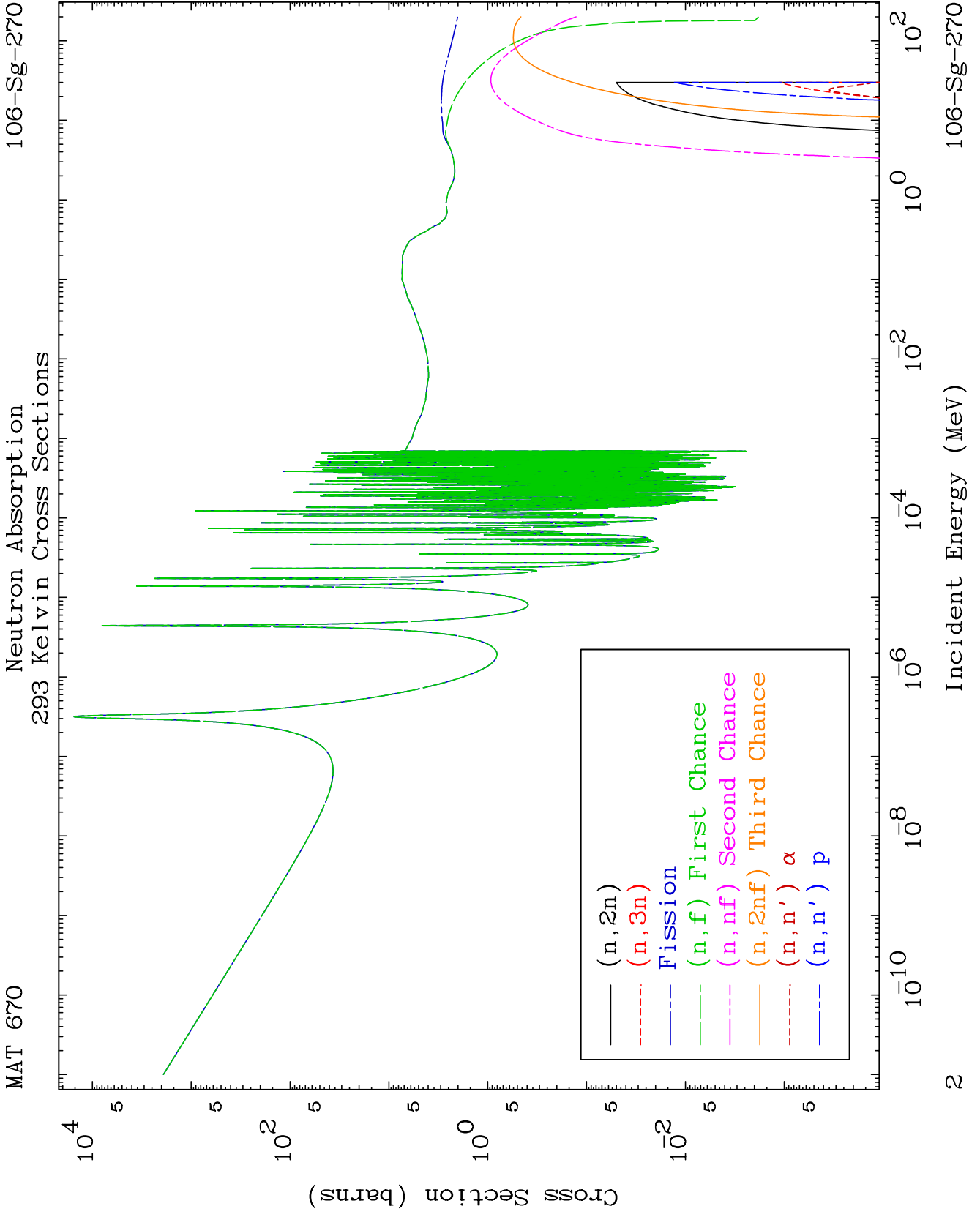
Press Mouse Button to Start

MAT 670

Neutron Major
293 Kelvin Cross Sections

106-Sg-270

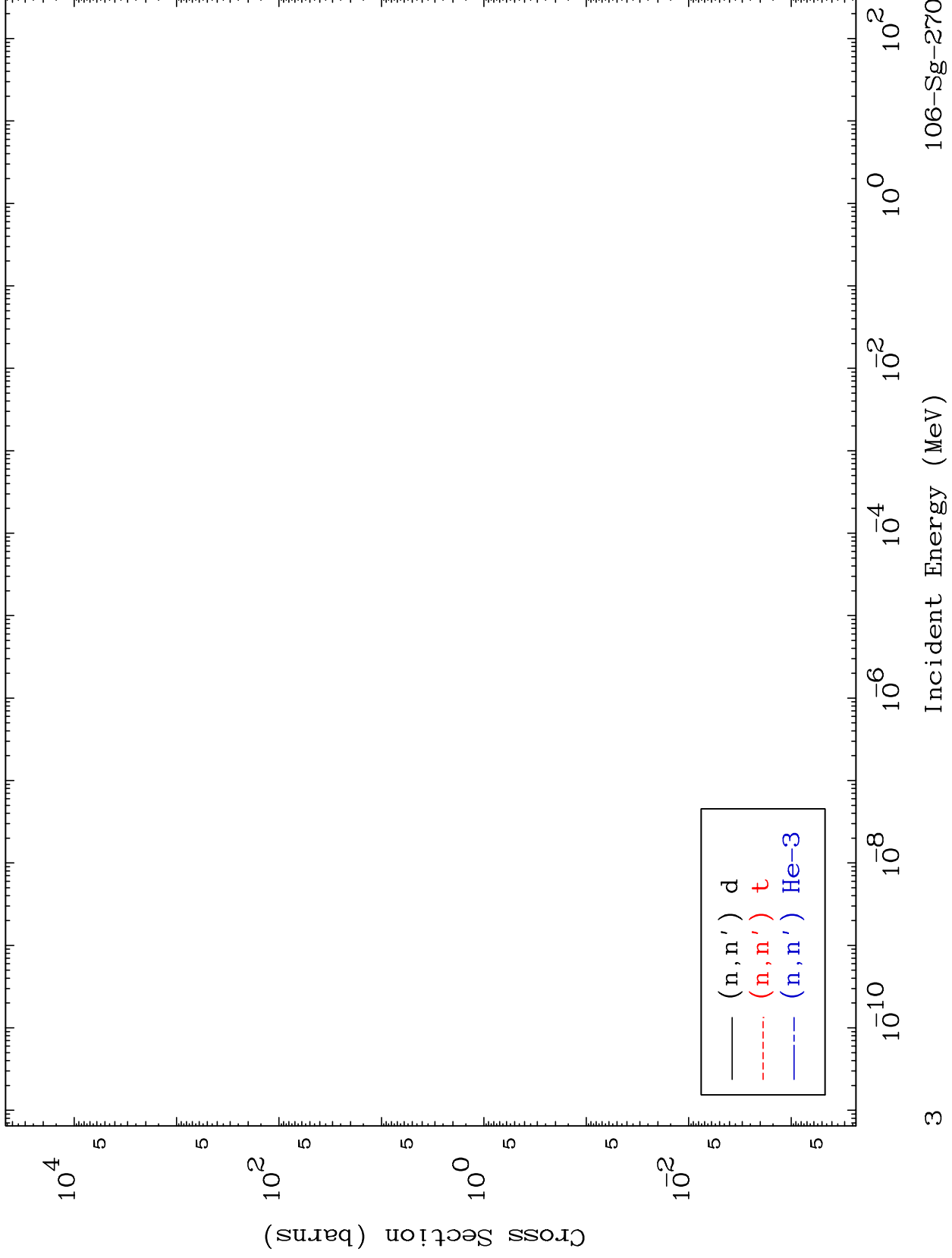




MAT 670

Neutron Absorption
293 Kelvin Cross Sections

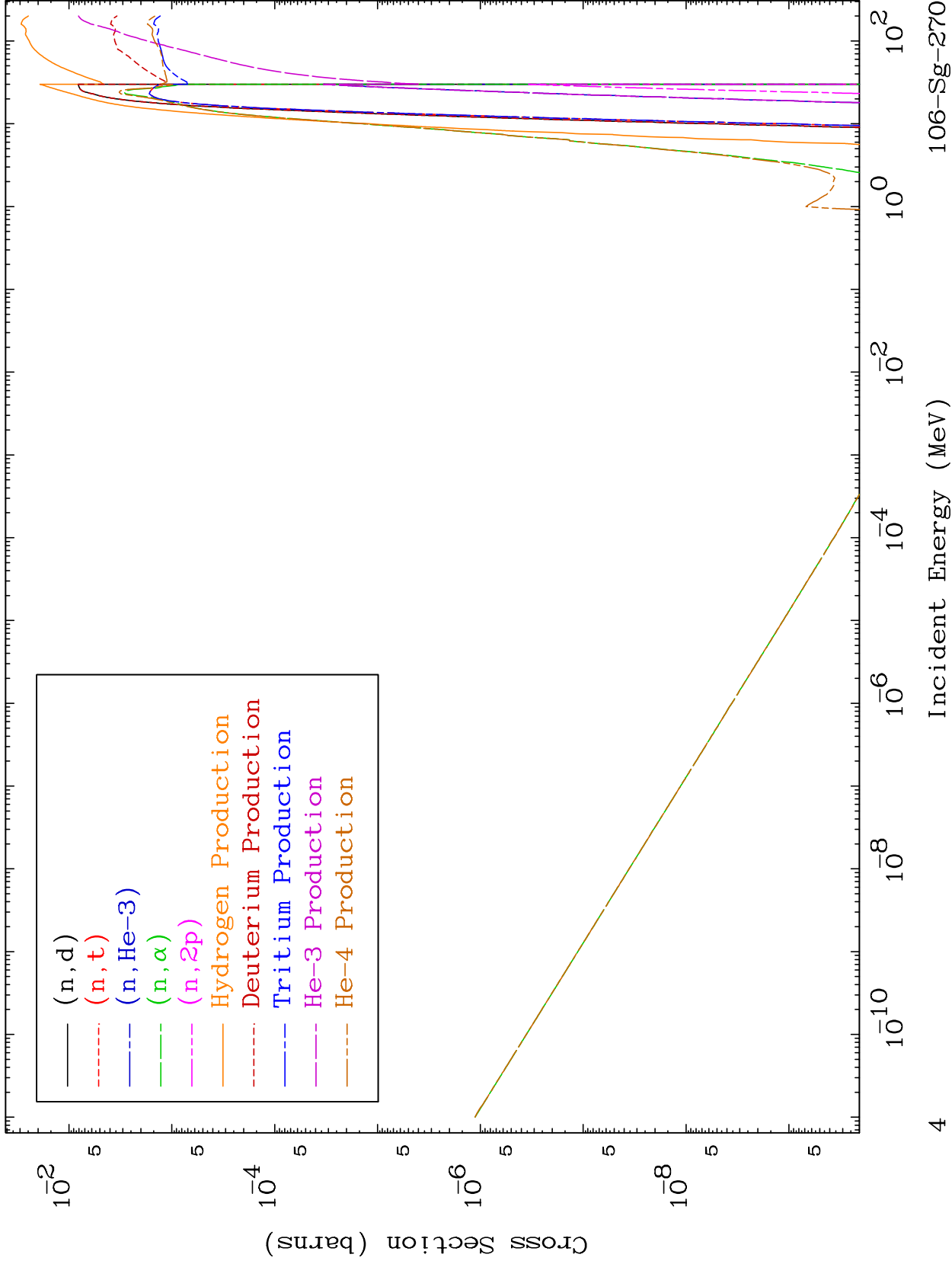
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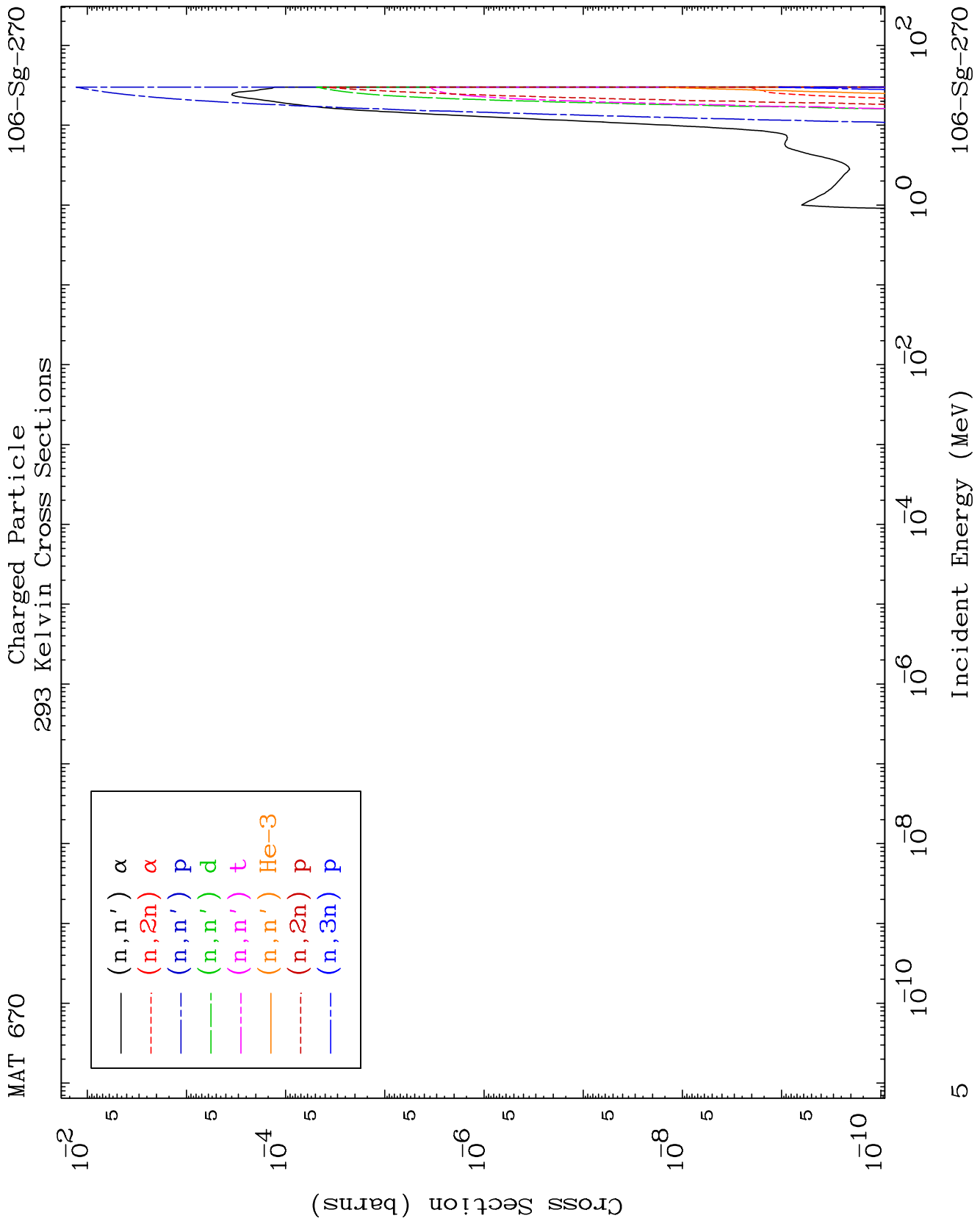


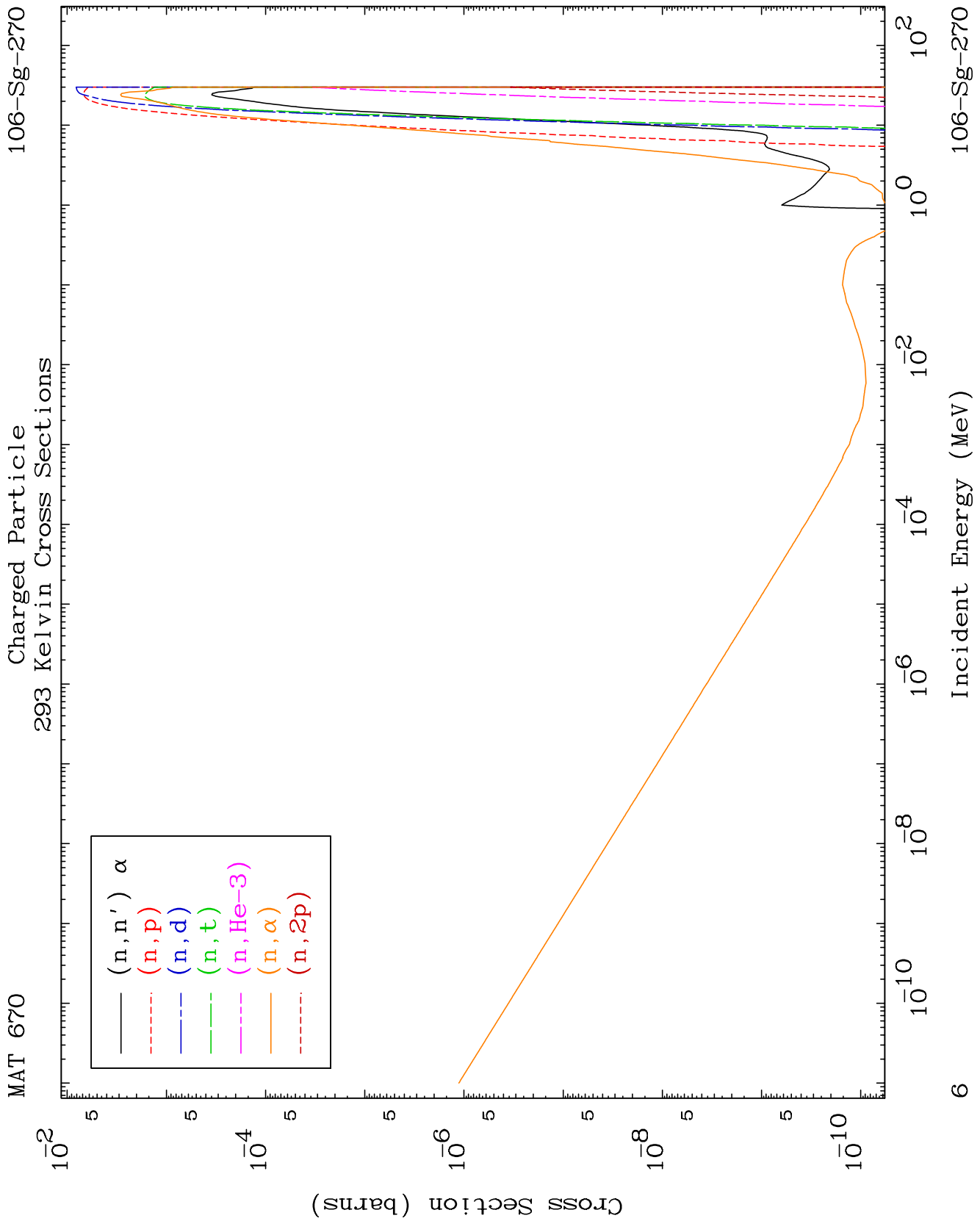
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Neutron Absorption
293 Kelvin Cross Sections

106-Sg-270



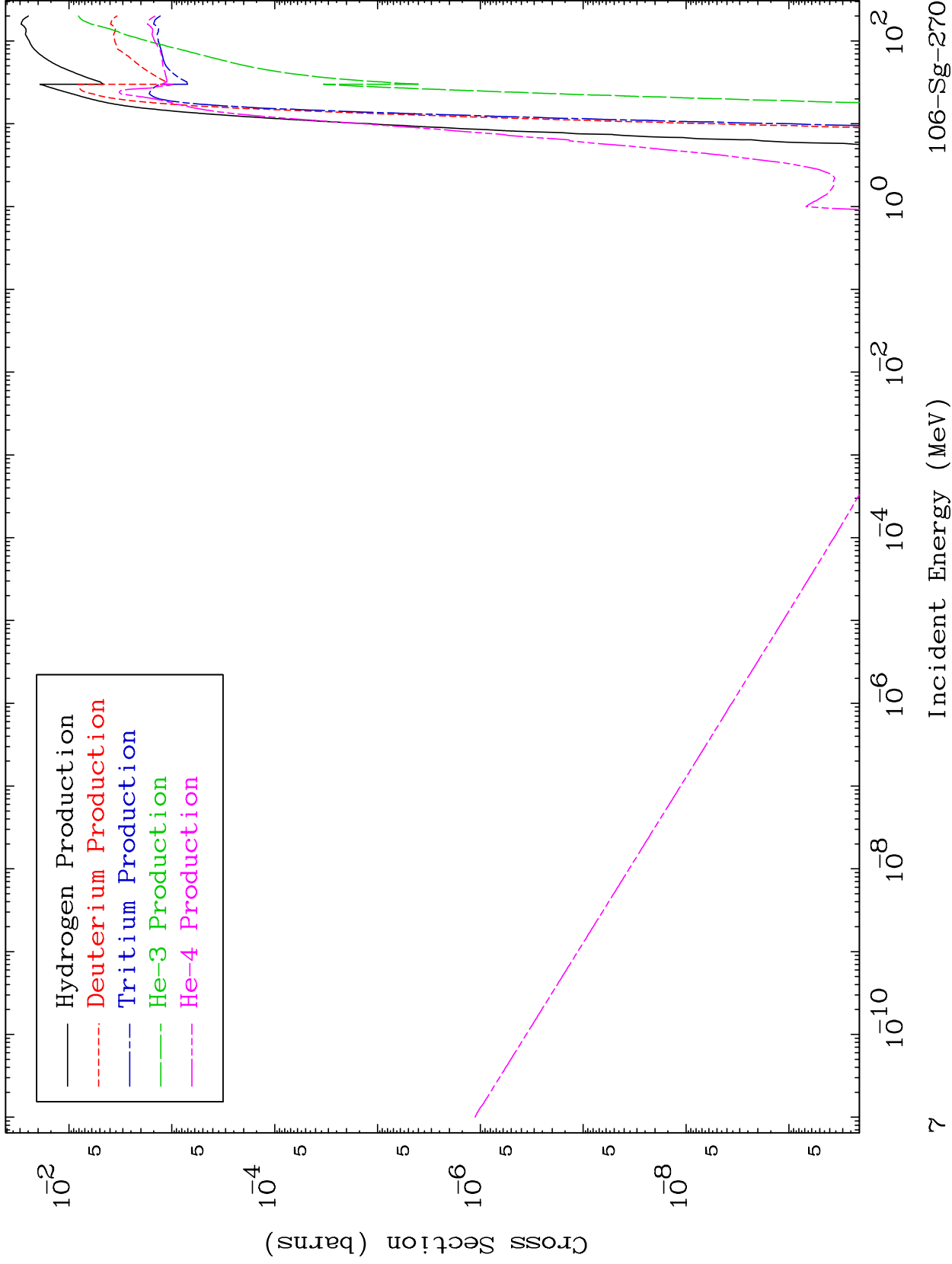




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Particle Production
293 Kelvin Cross Sections

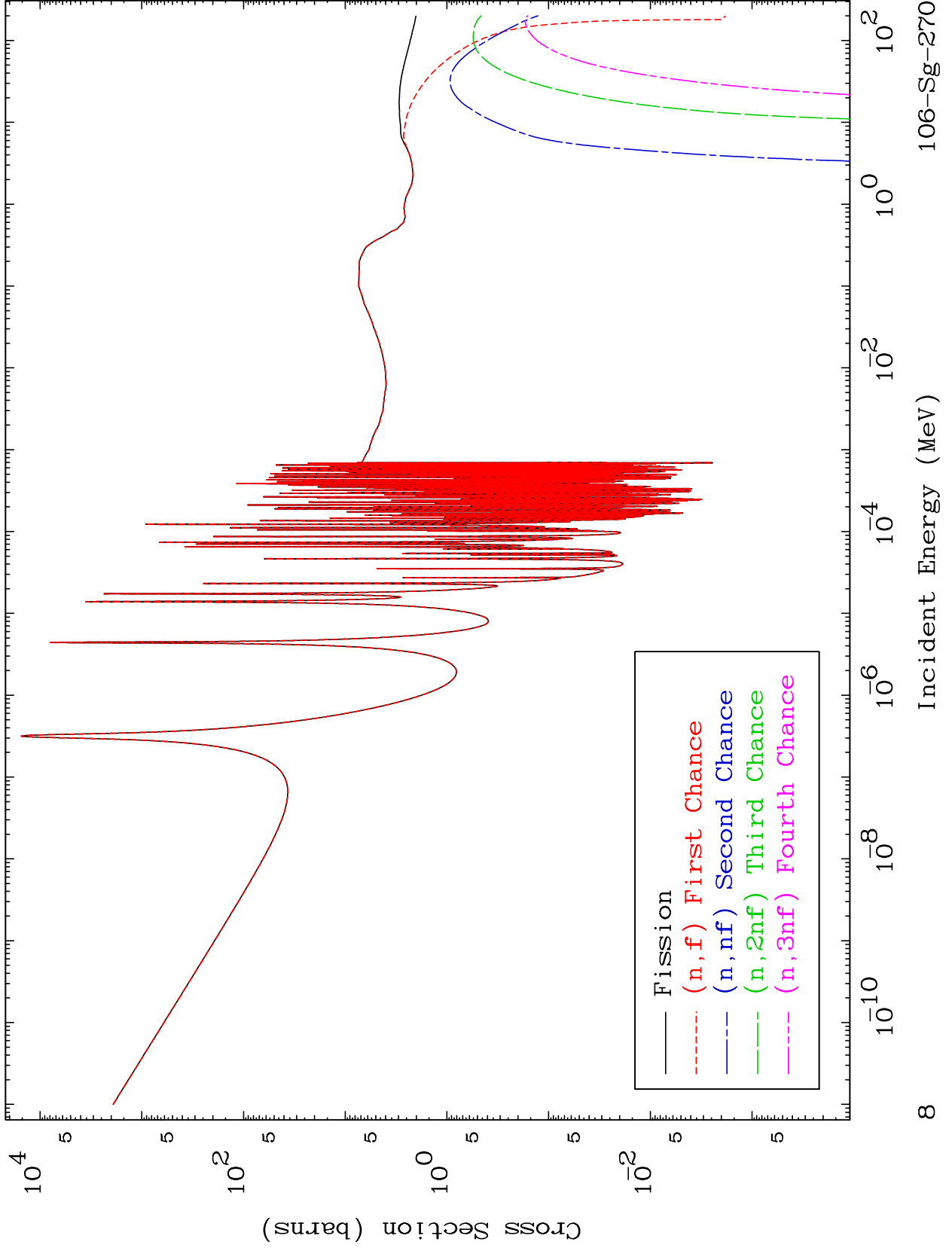
106-Sg-270



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Fission
293 Kelvin Cross Sections

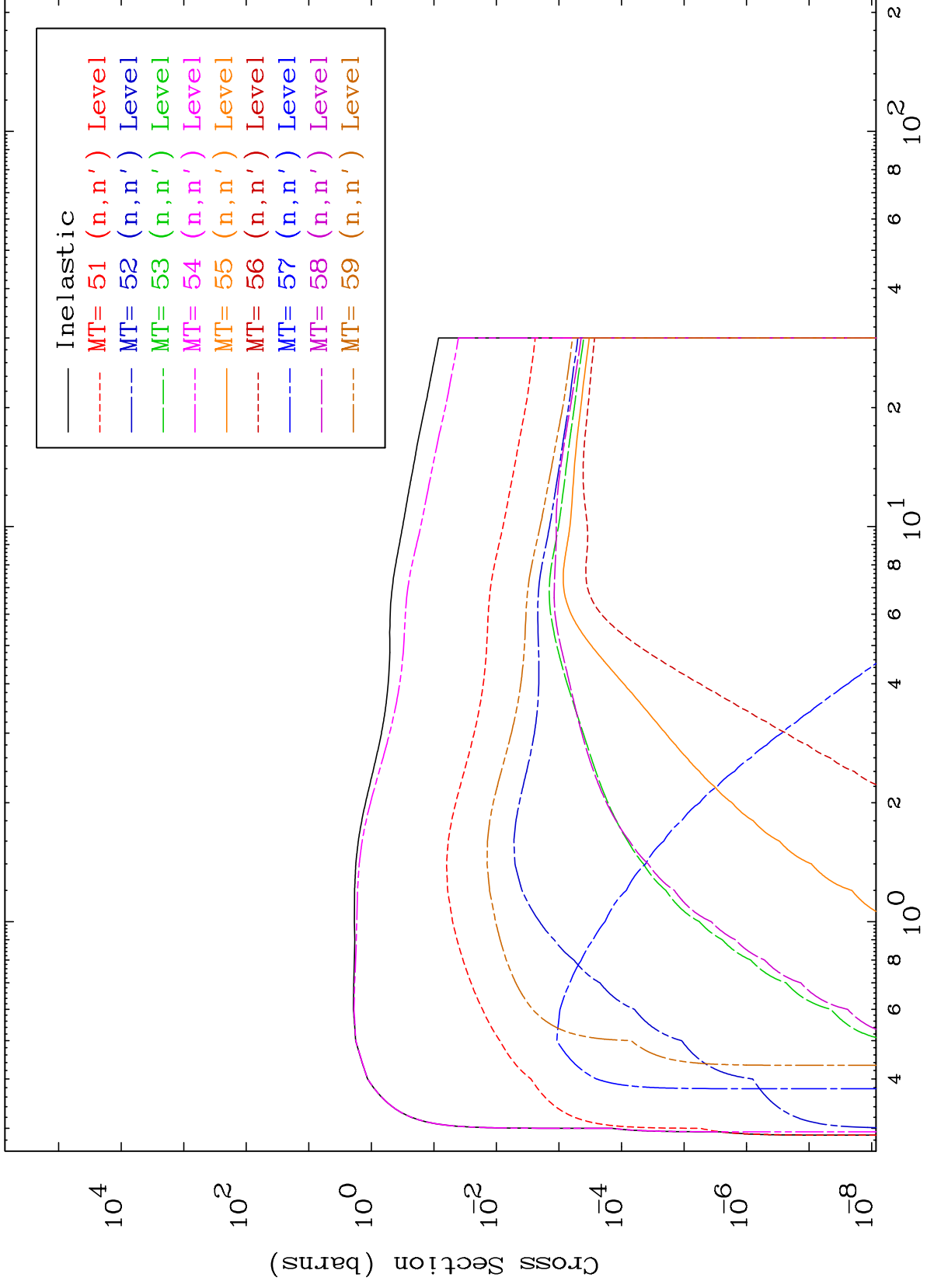
106-Sg-270



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(n,n') Levels
293 Kelvin Cross Sections

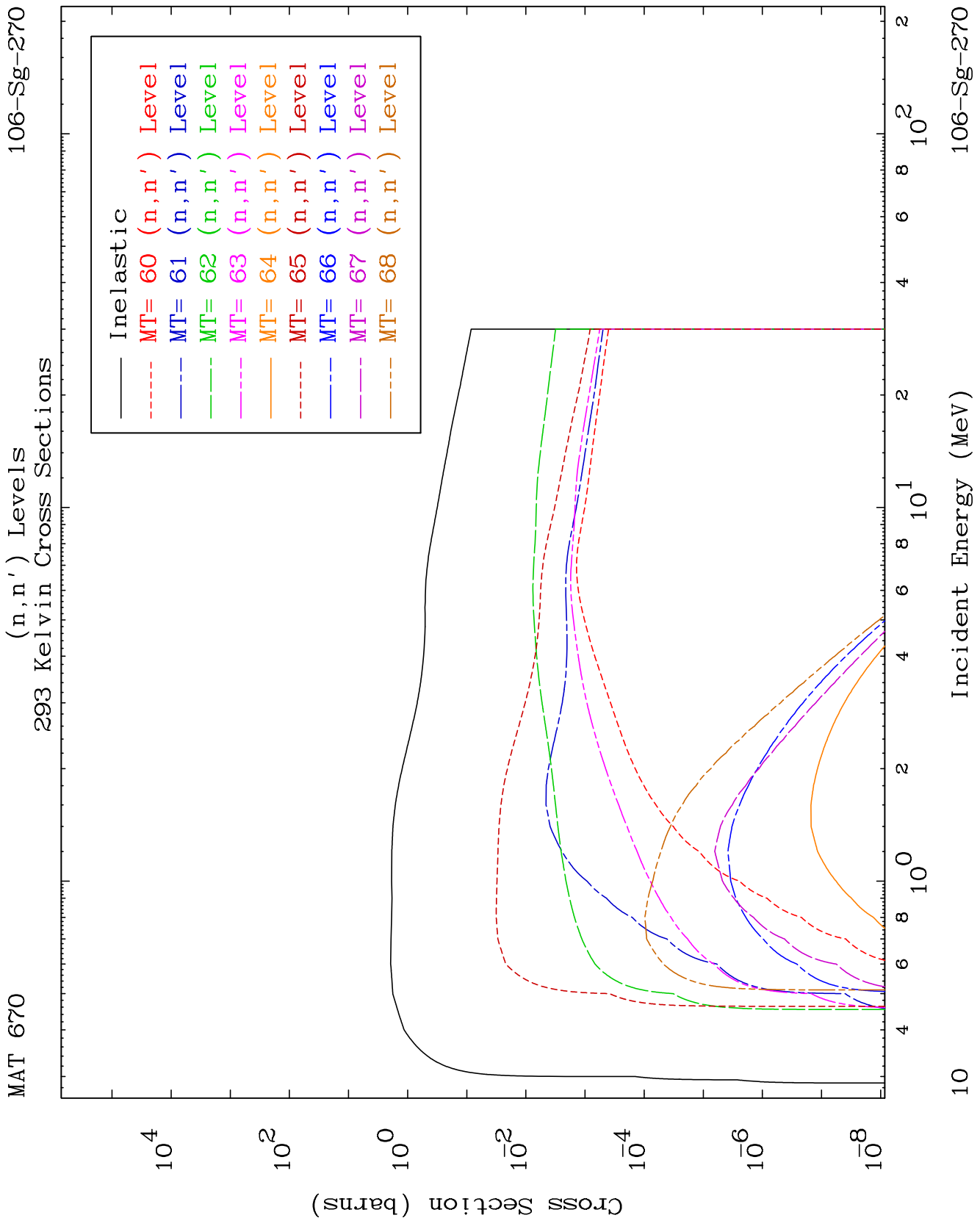
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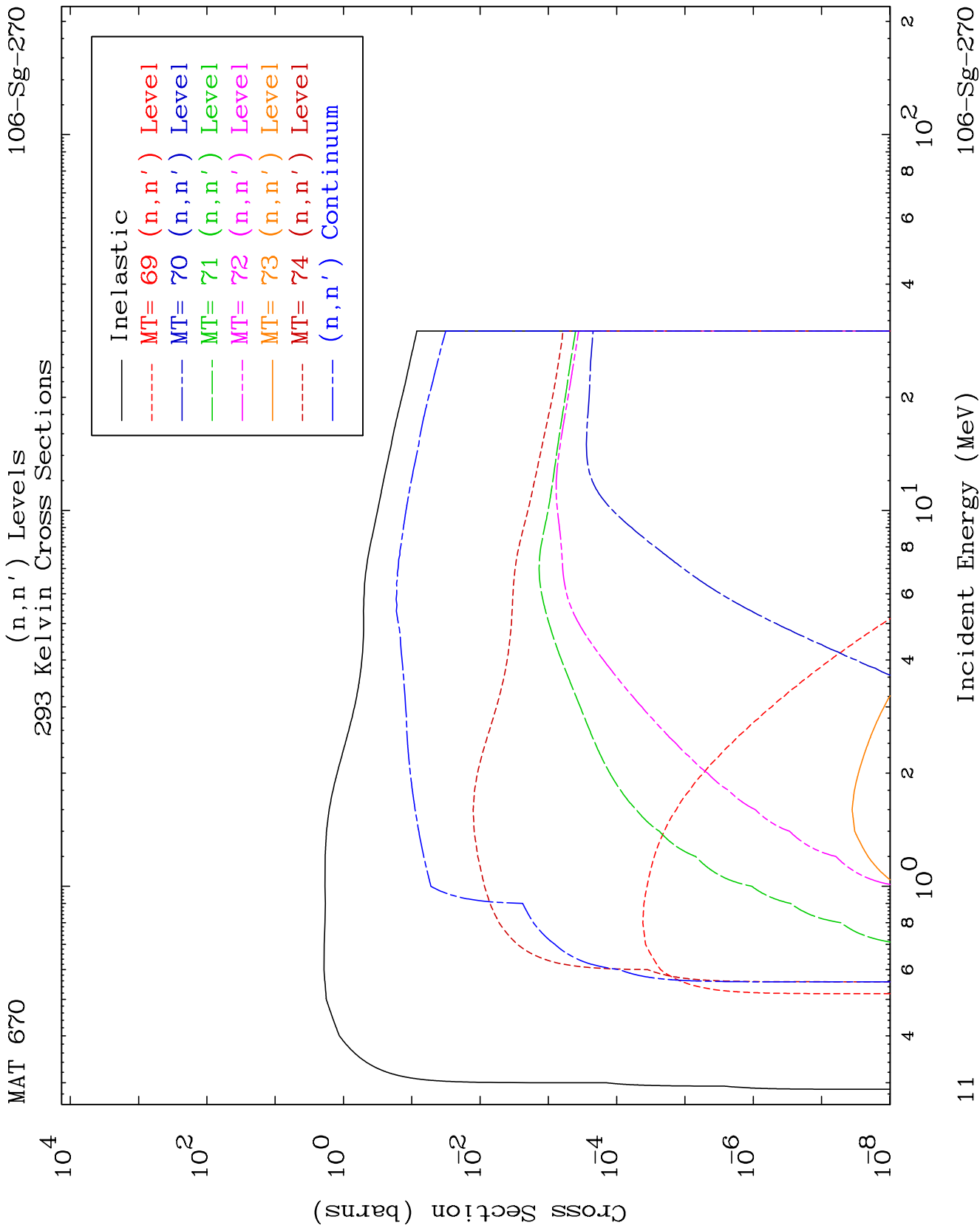


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Incident Energy (MeV)

106-Sg-270

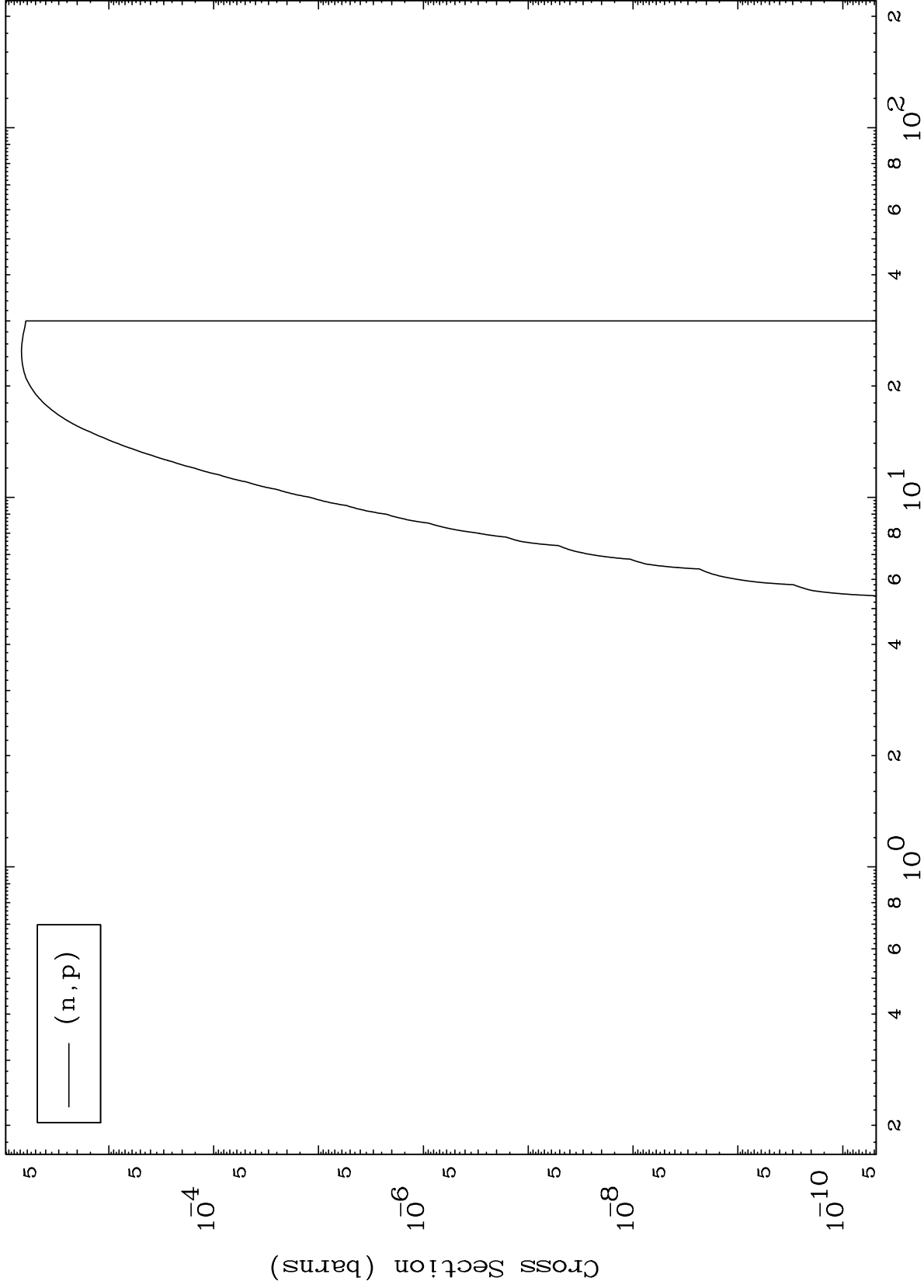




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(n,p) Levels
293 Kelvin Cross Sections

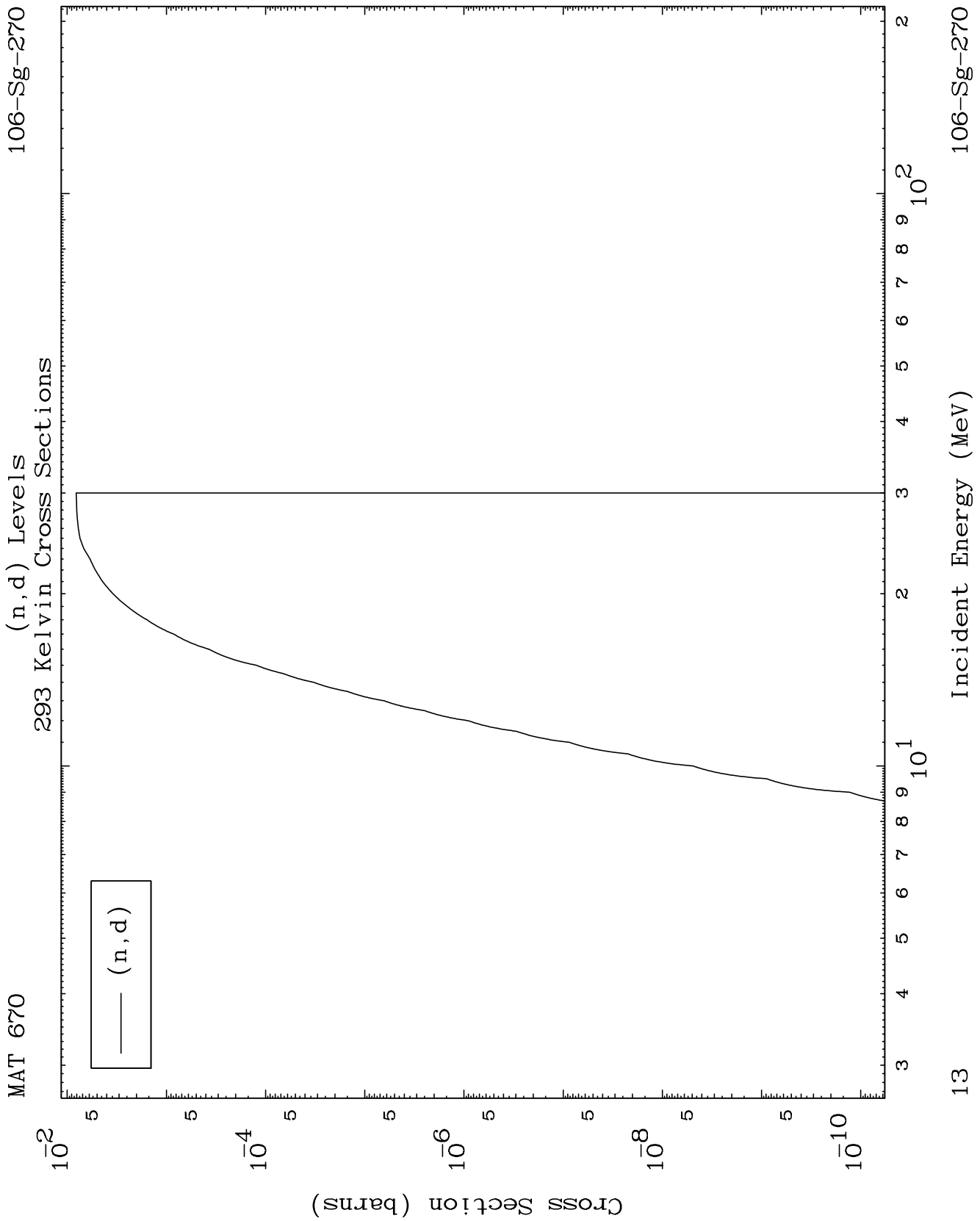
106-Sg-270



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Incident Energy (MeV)

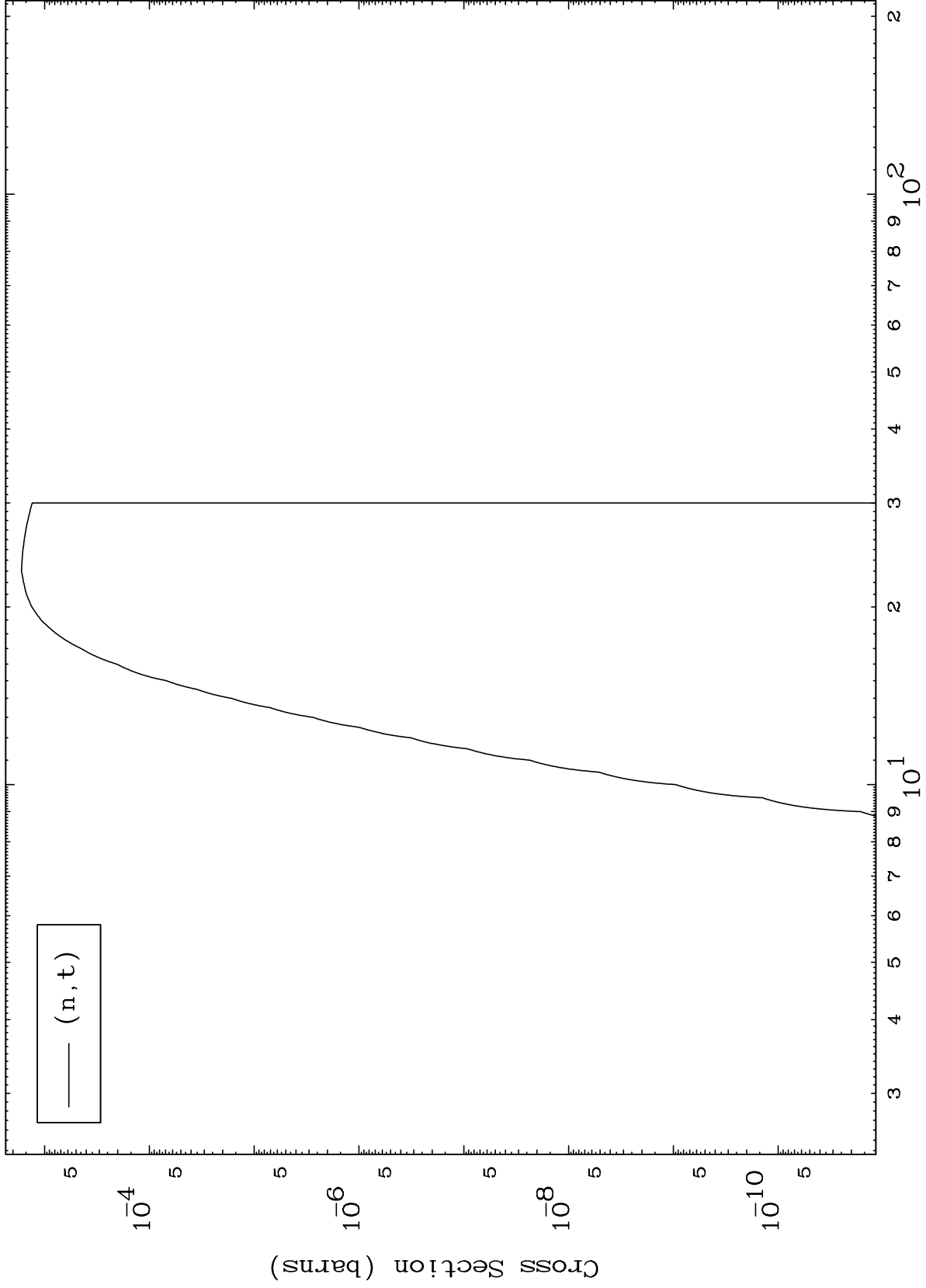
106-Sg-270



MAT 670

(n,t) Levels
293 Kelvin Cross Sections

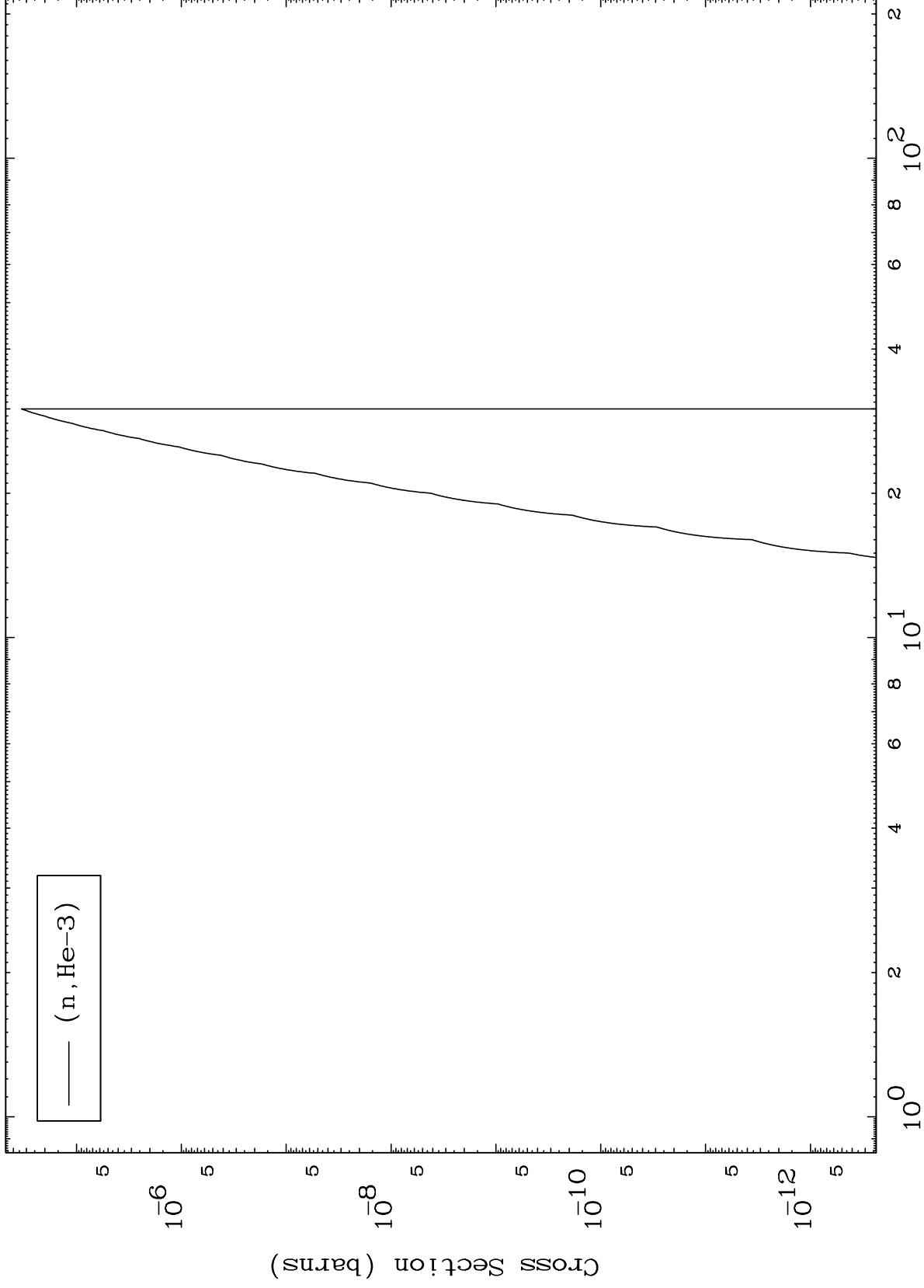
106-Sg-270



MAT 670

(n,He3) Levels
293 Kelvin Cross Sections

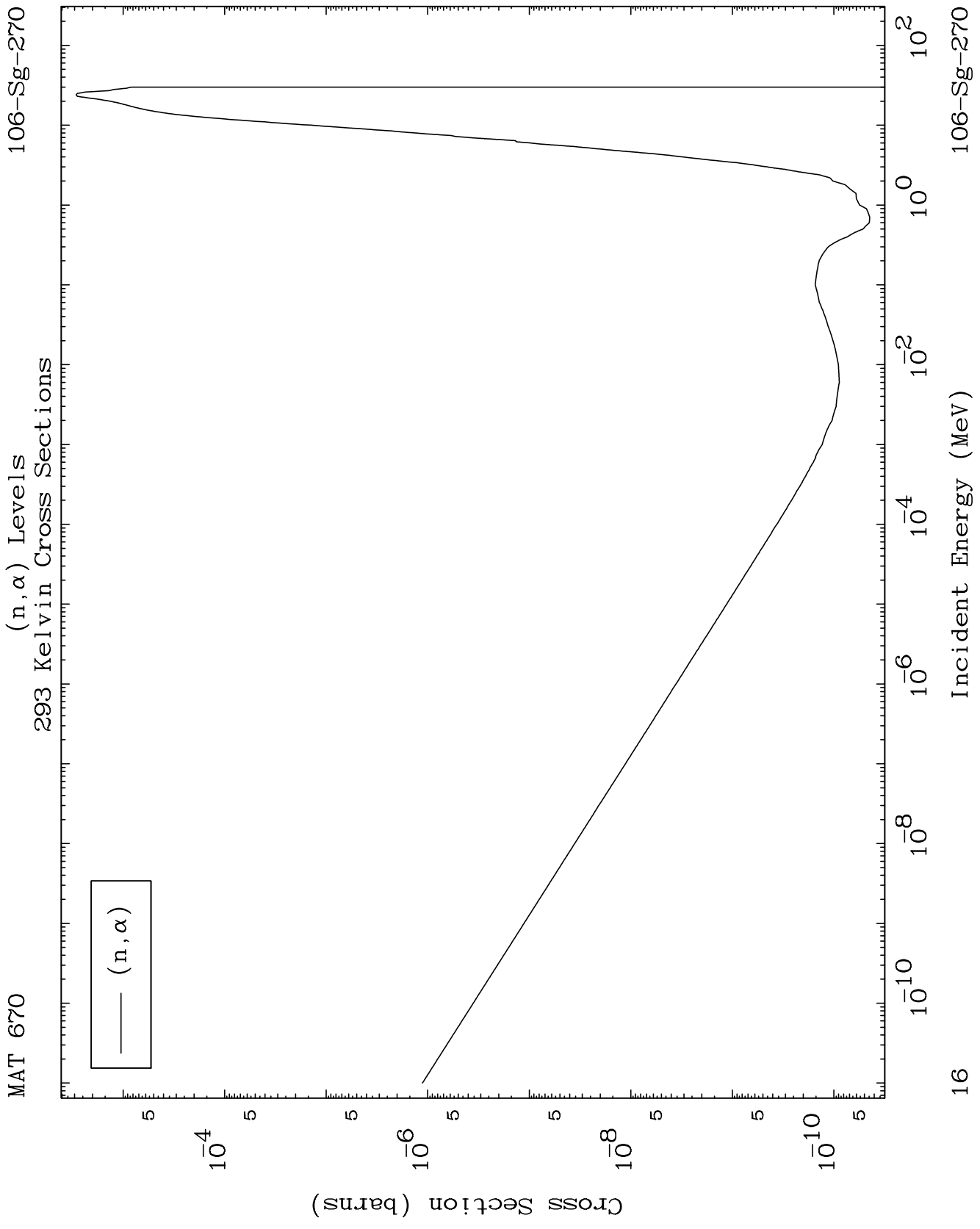
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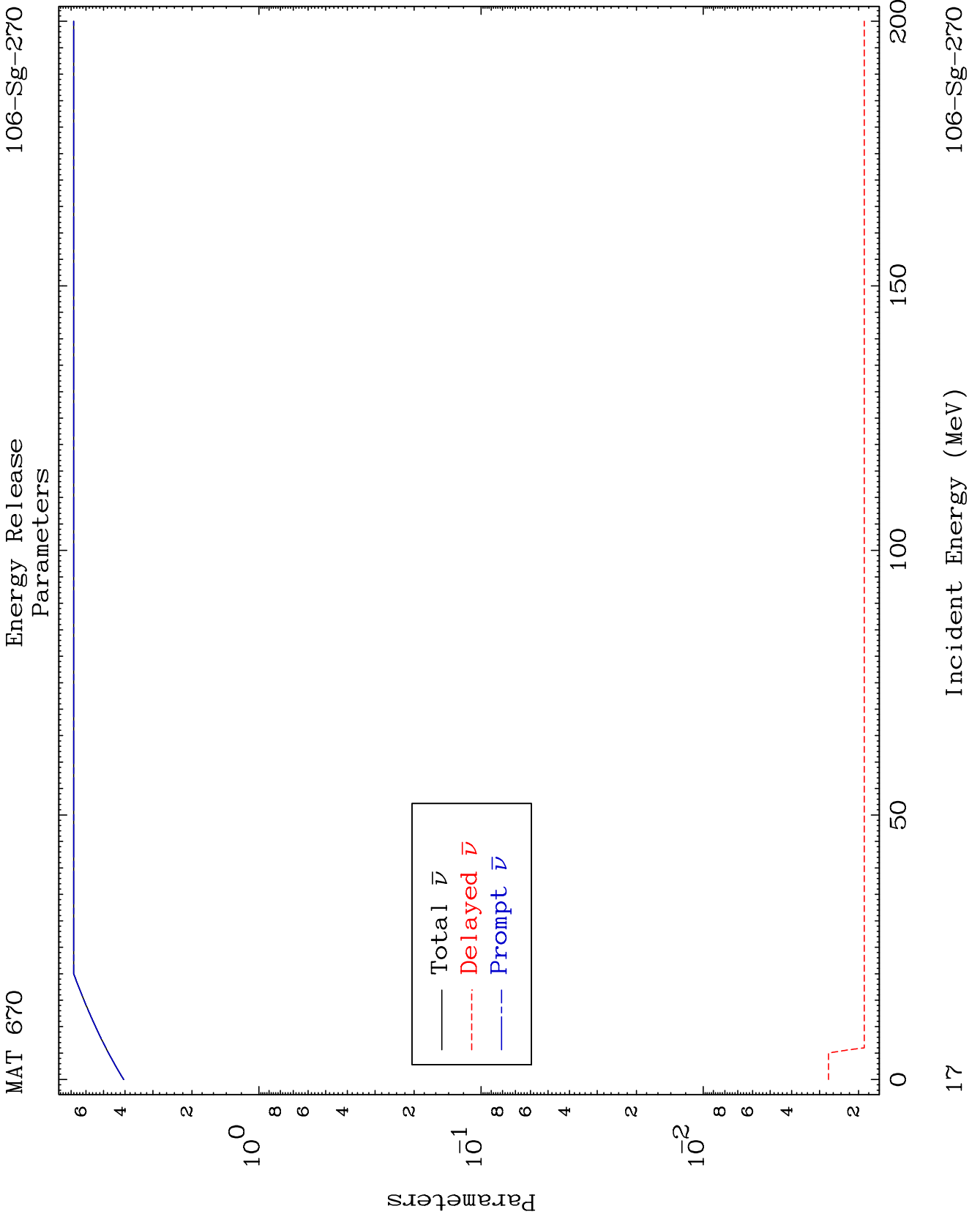


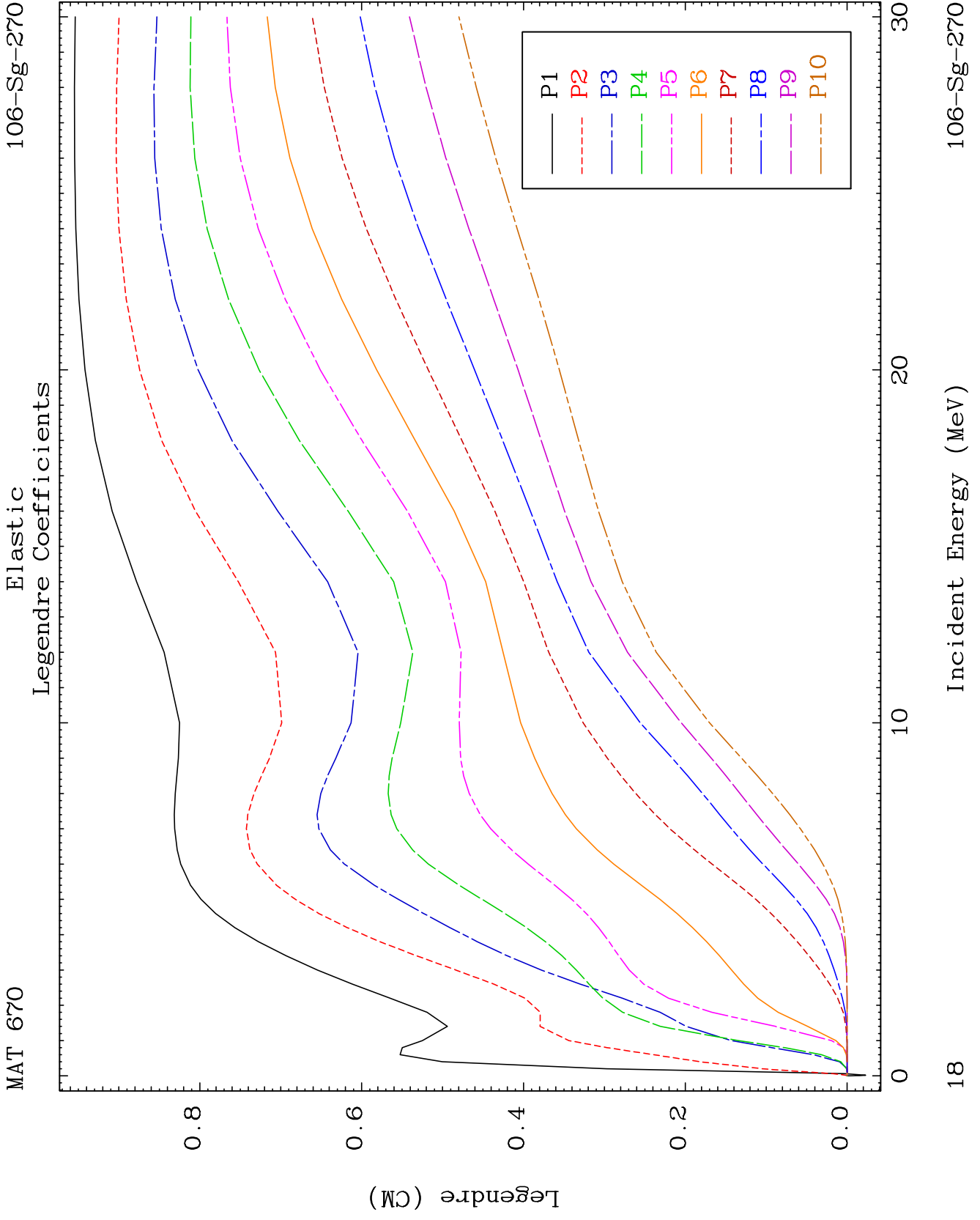
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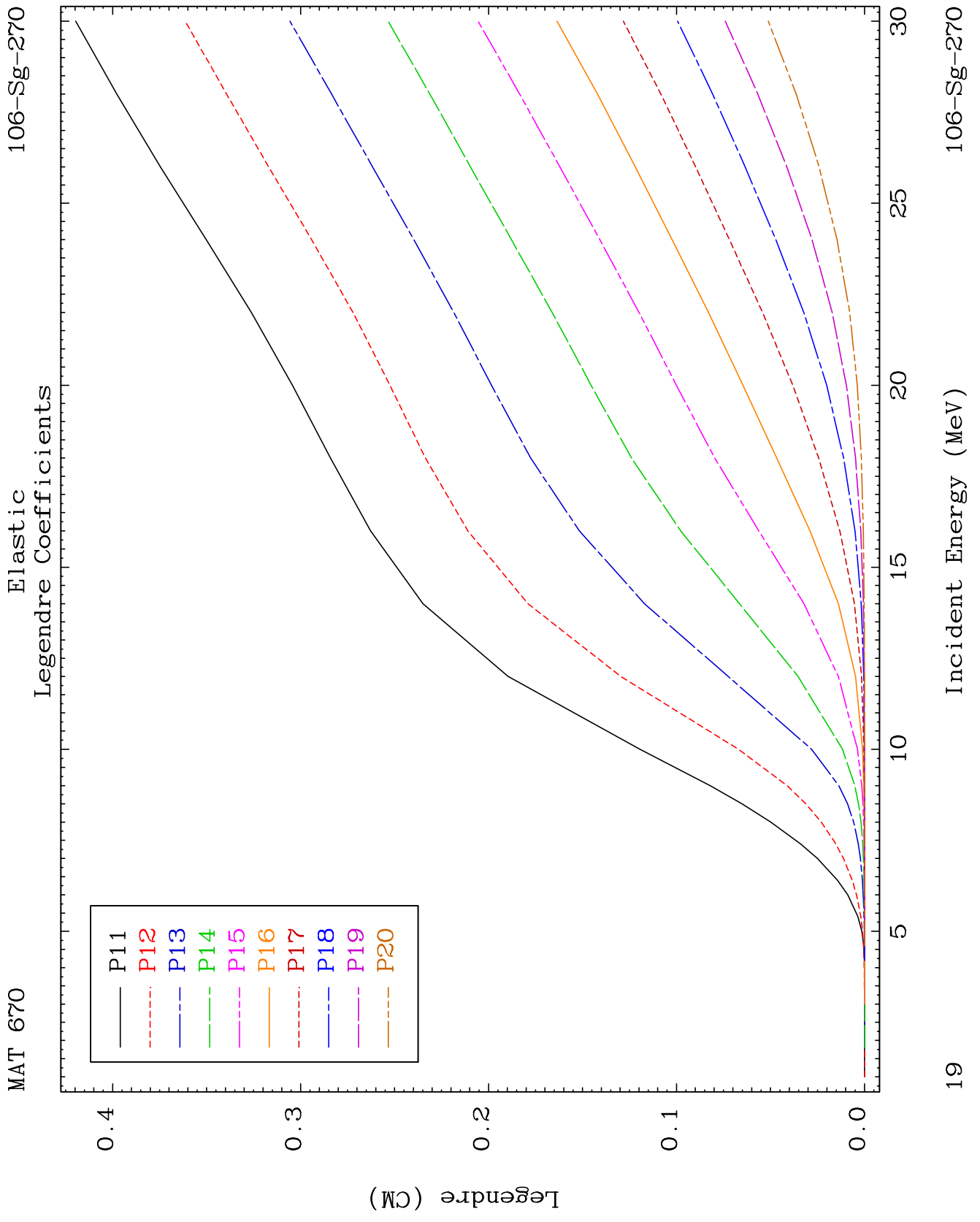
Incident Energy (MeV)

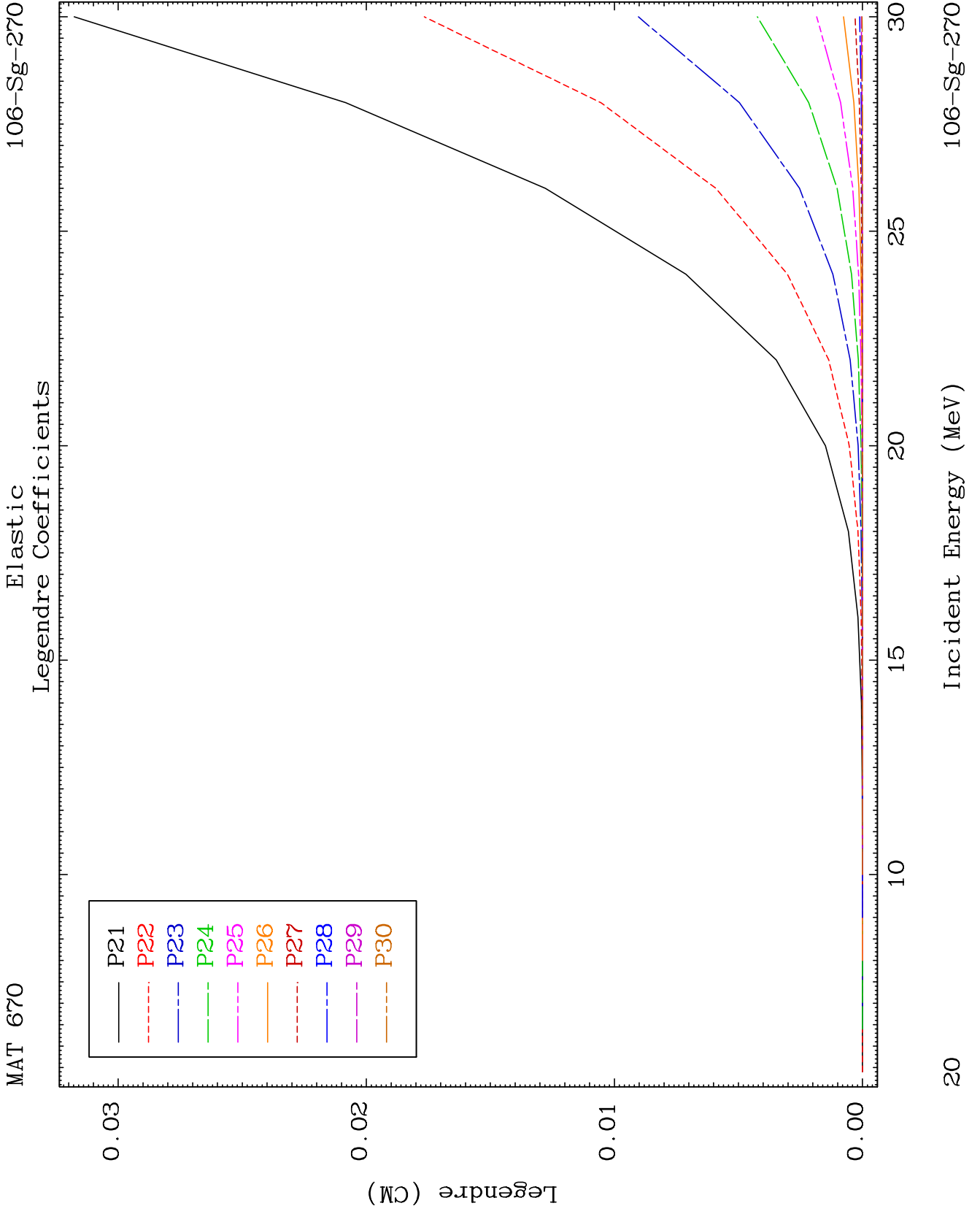
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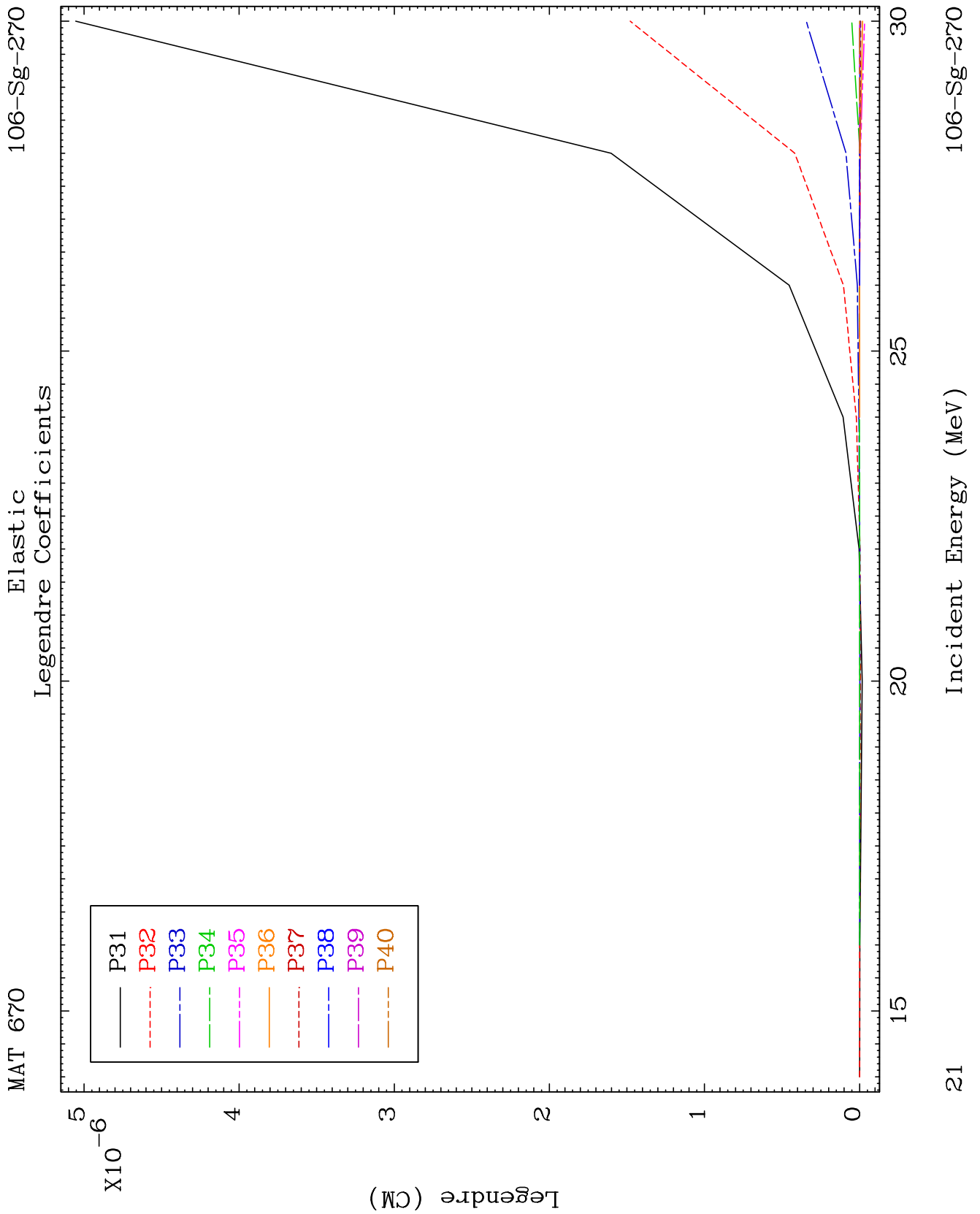








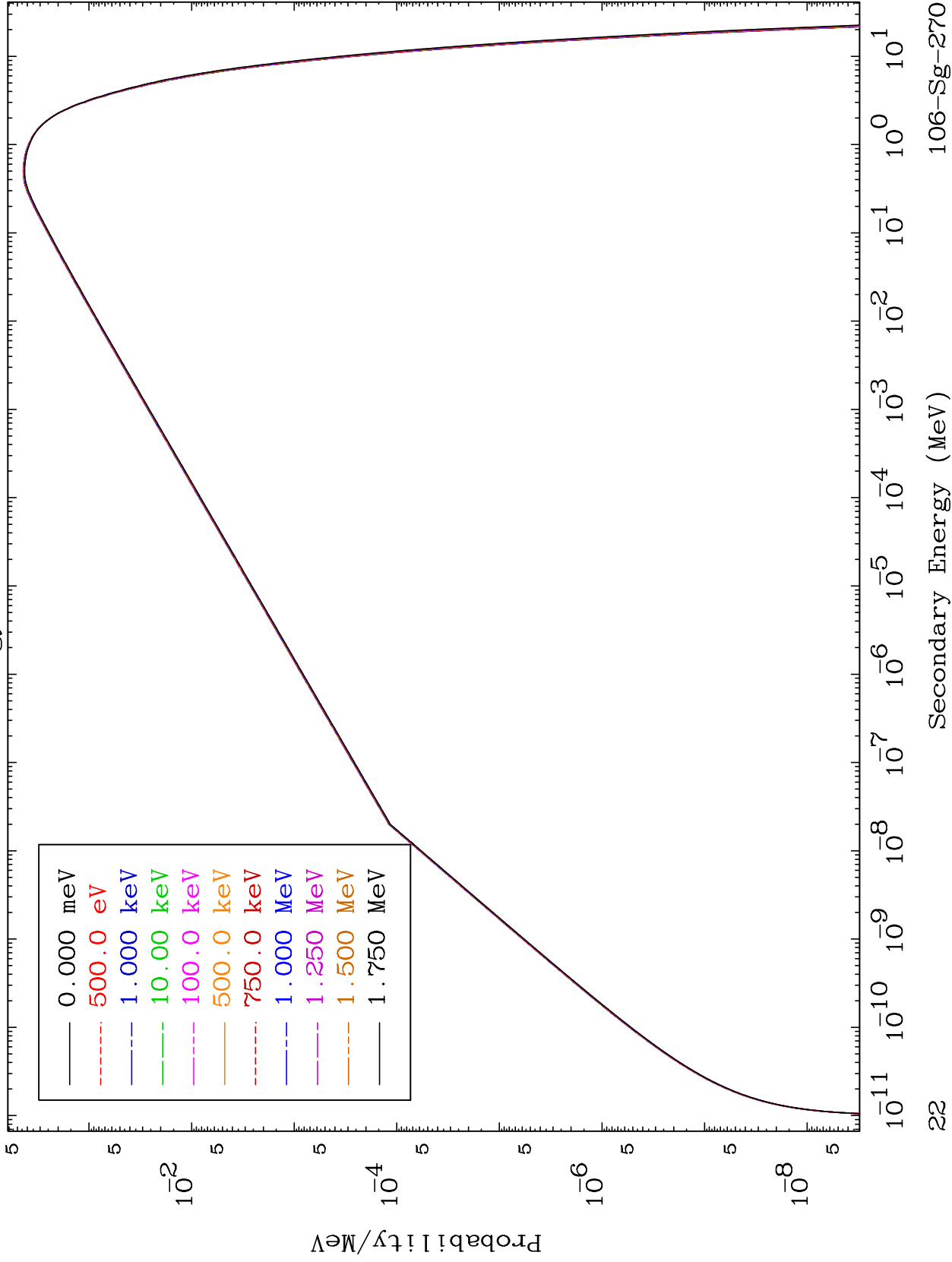




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Fission Energy Distributions

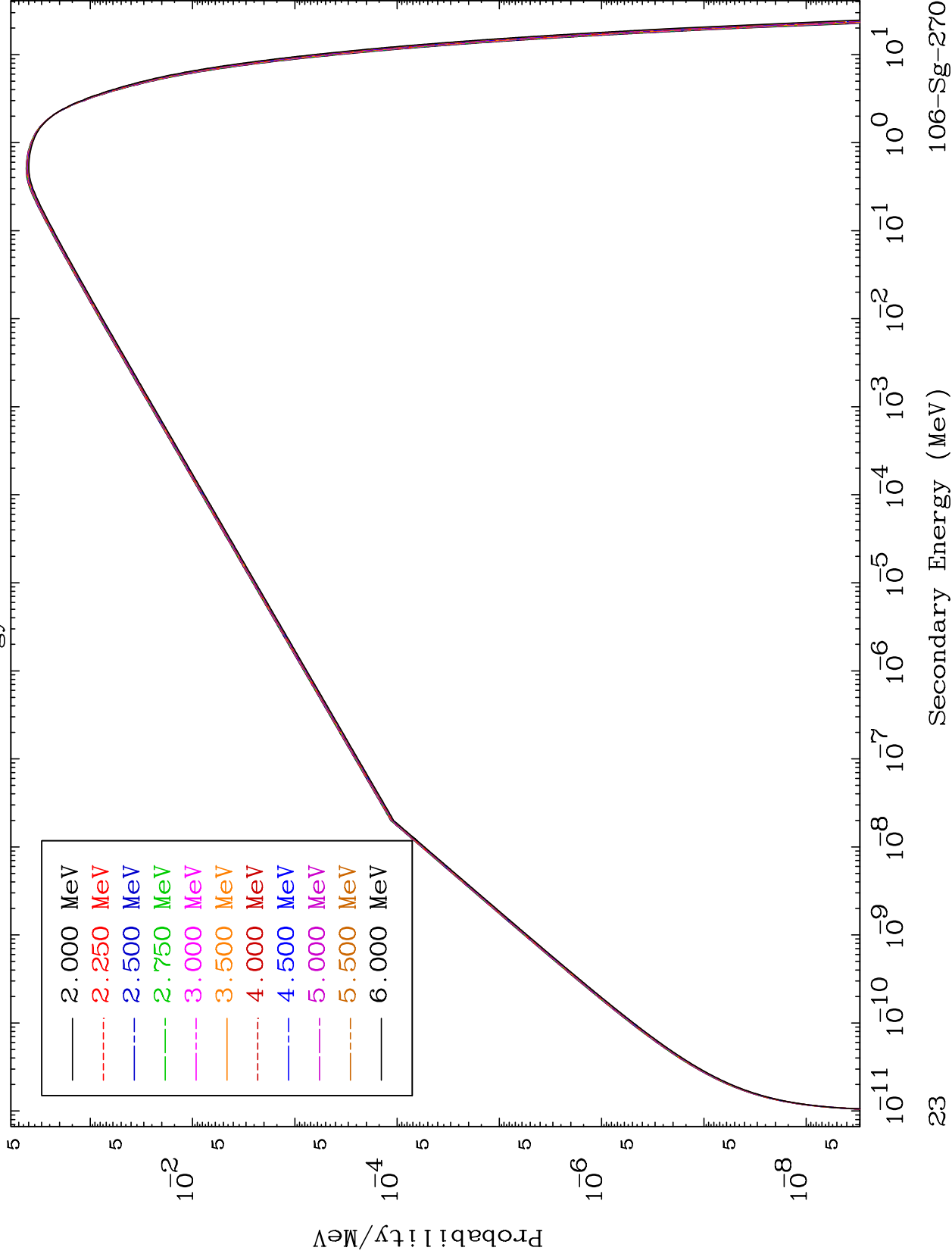
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Fission Energy Distributions

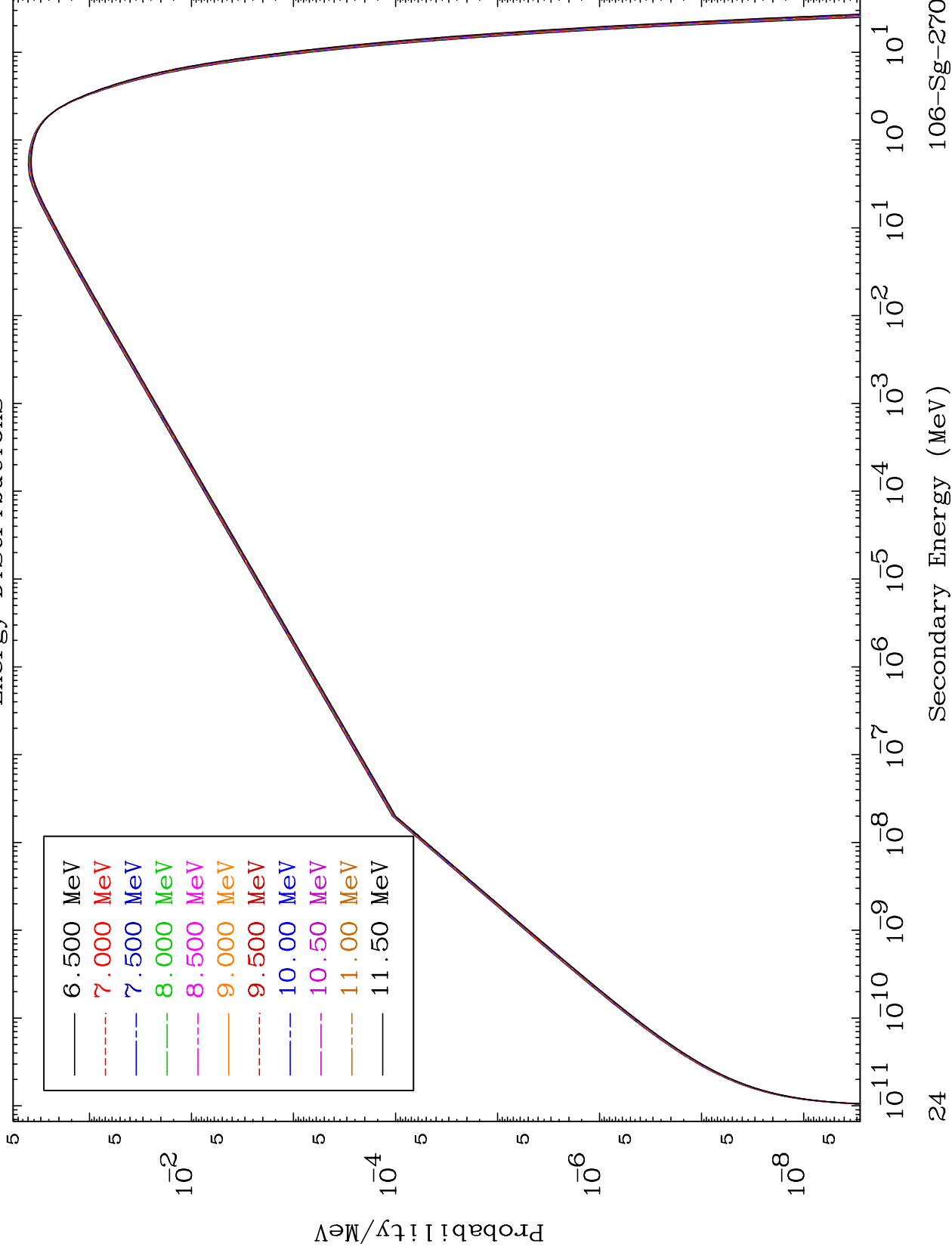
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Fission Energy Distributions

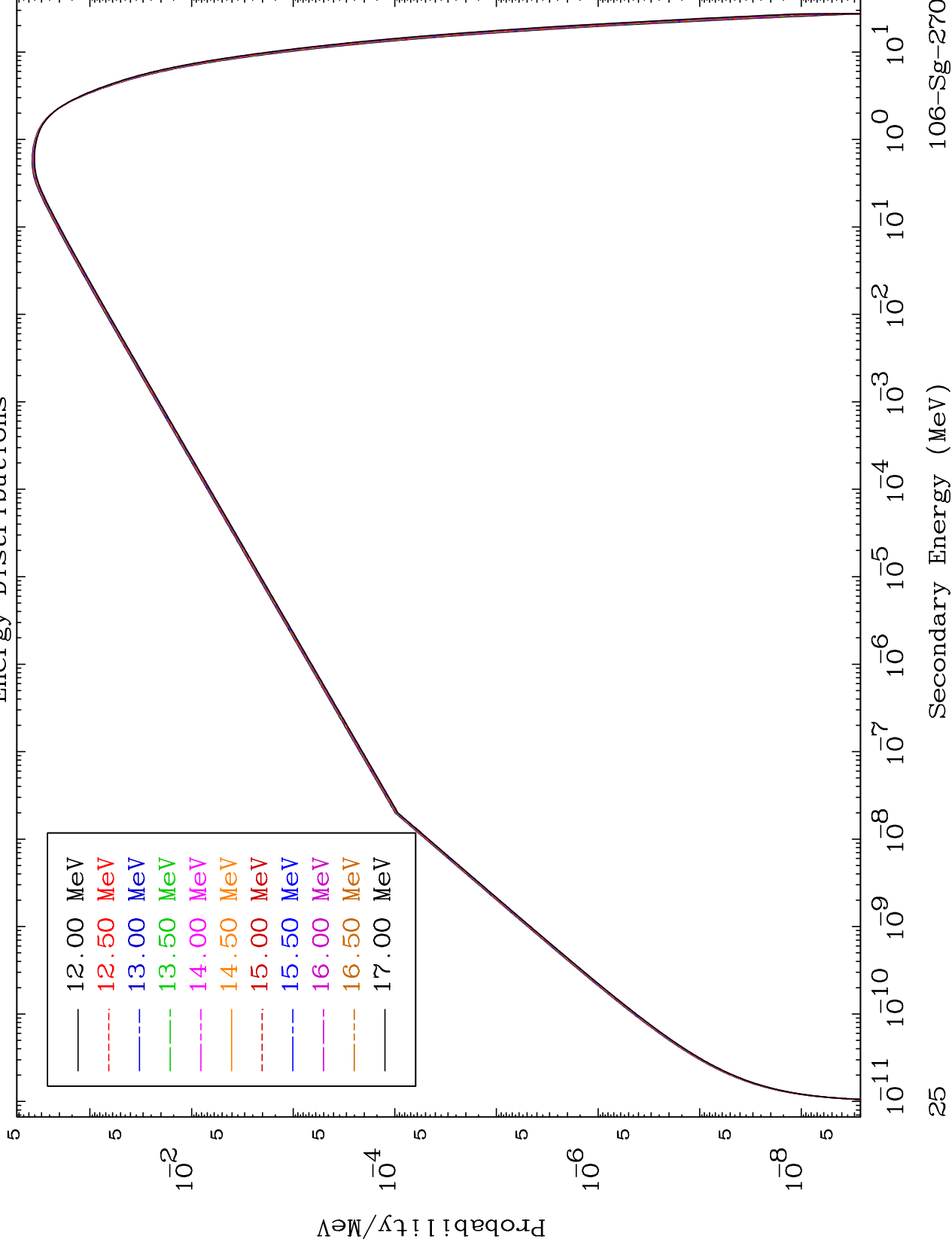
106-Sg-270



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Fission Energy Distributions

106-Sg-270



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106-Sg-270

MAT 670

Fission Energy Distributions

106-Sg-270

