

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

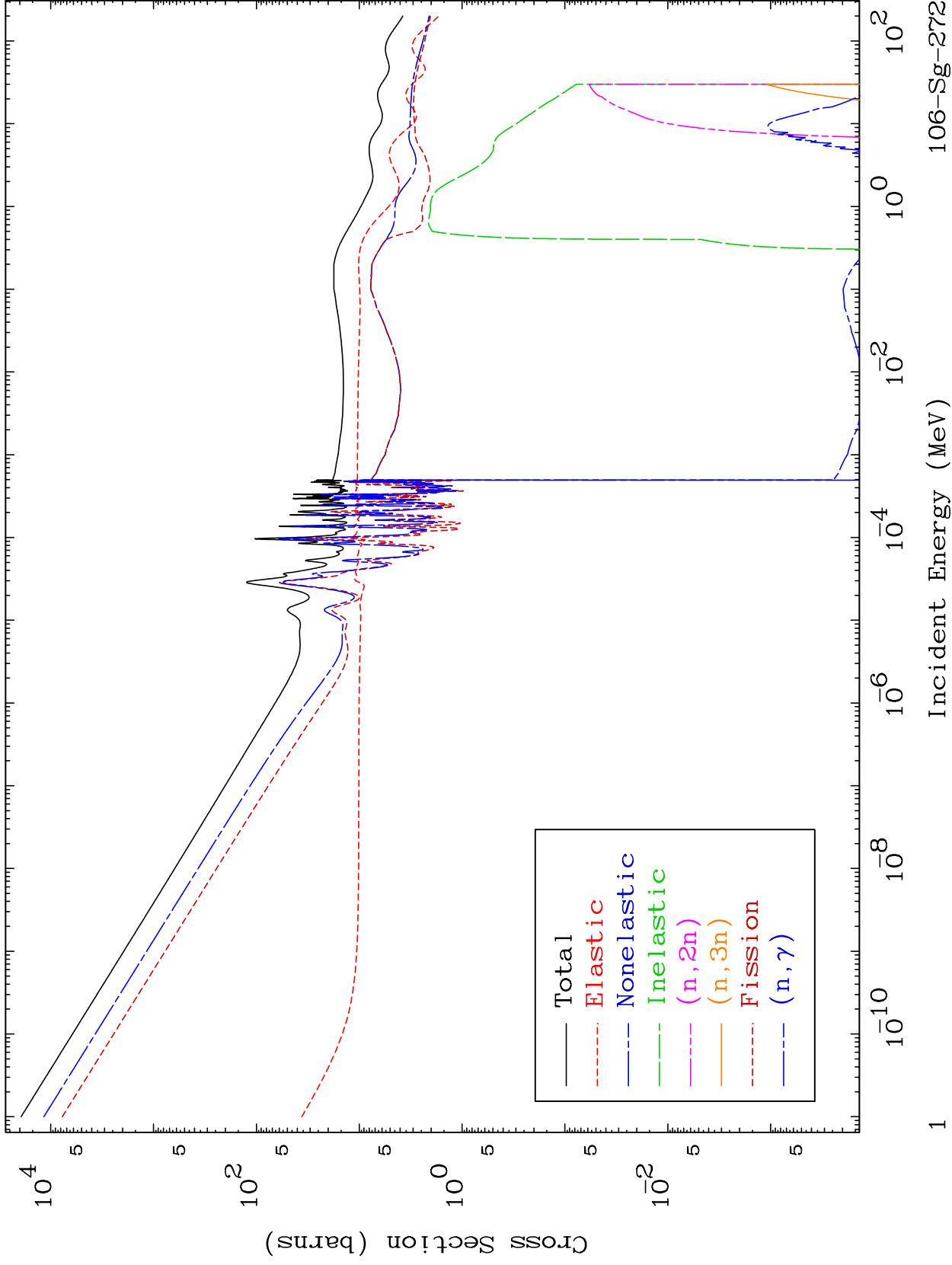
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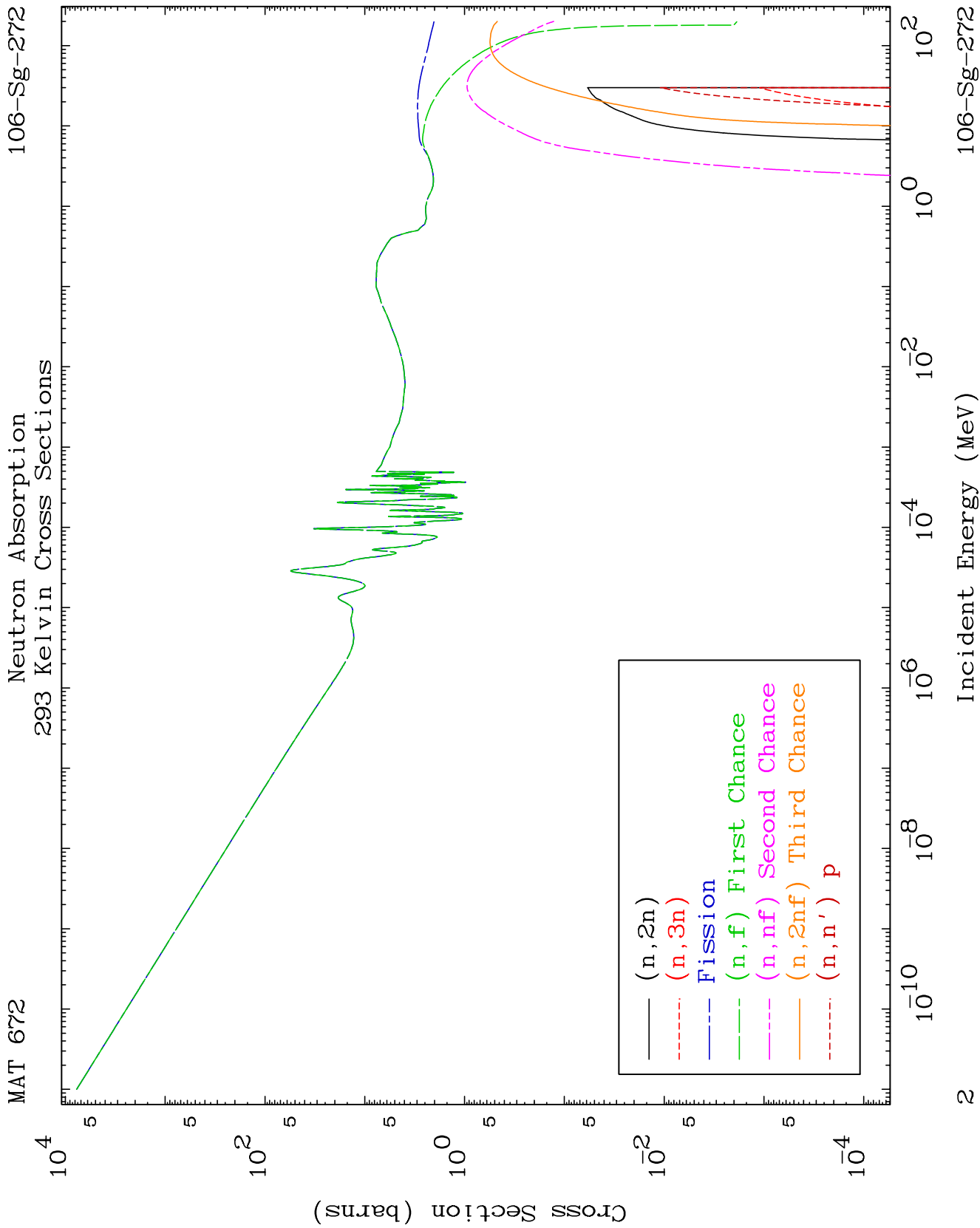
Press Mouse Button to Start

MAT 672

Neutron Major  
293 Kelvin Cross Sections

106-Sg-272

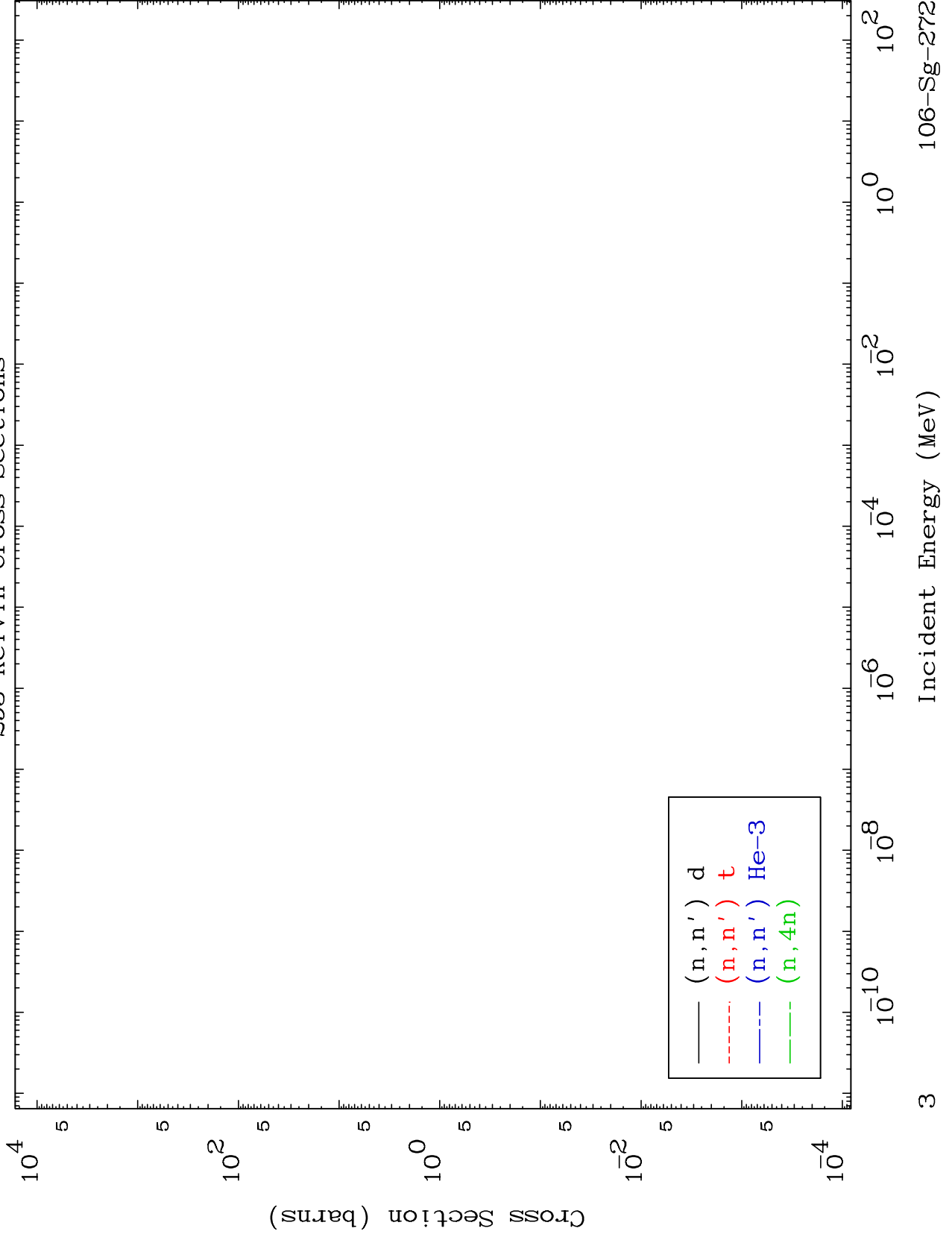




MAT 672

Neutron Absorption  
293 Kelvin Cross Sections

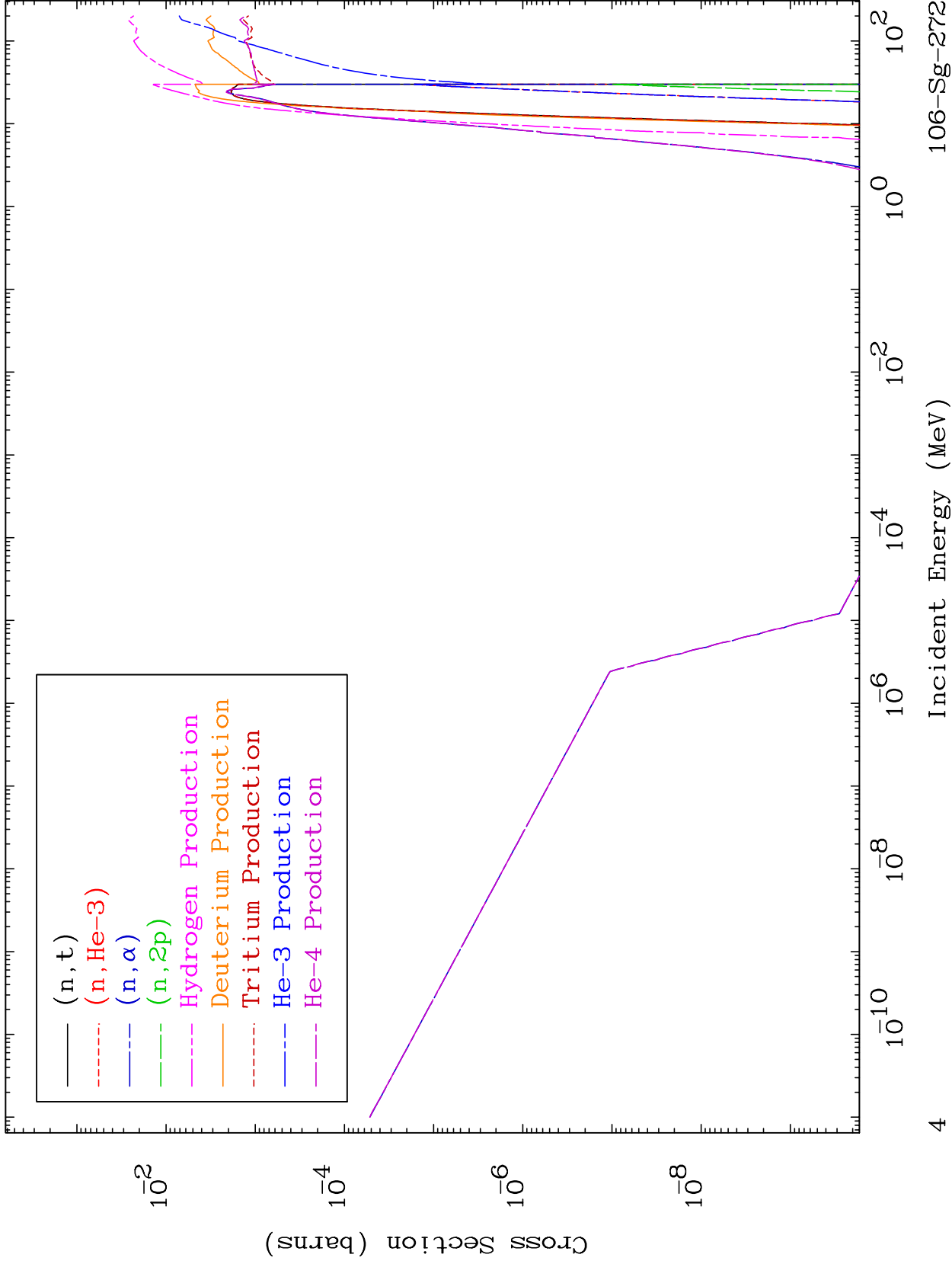
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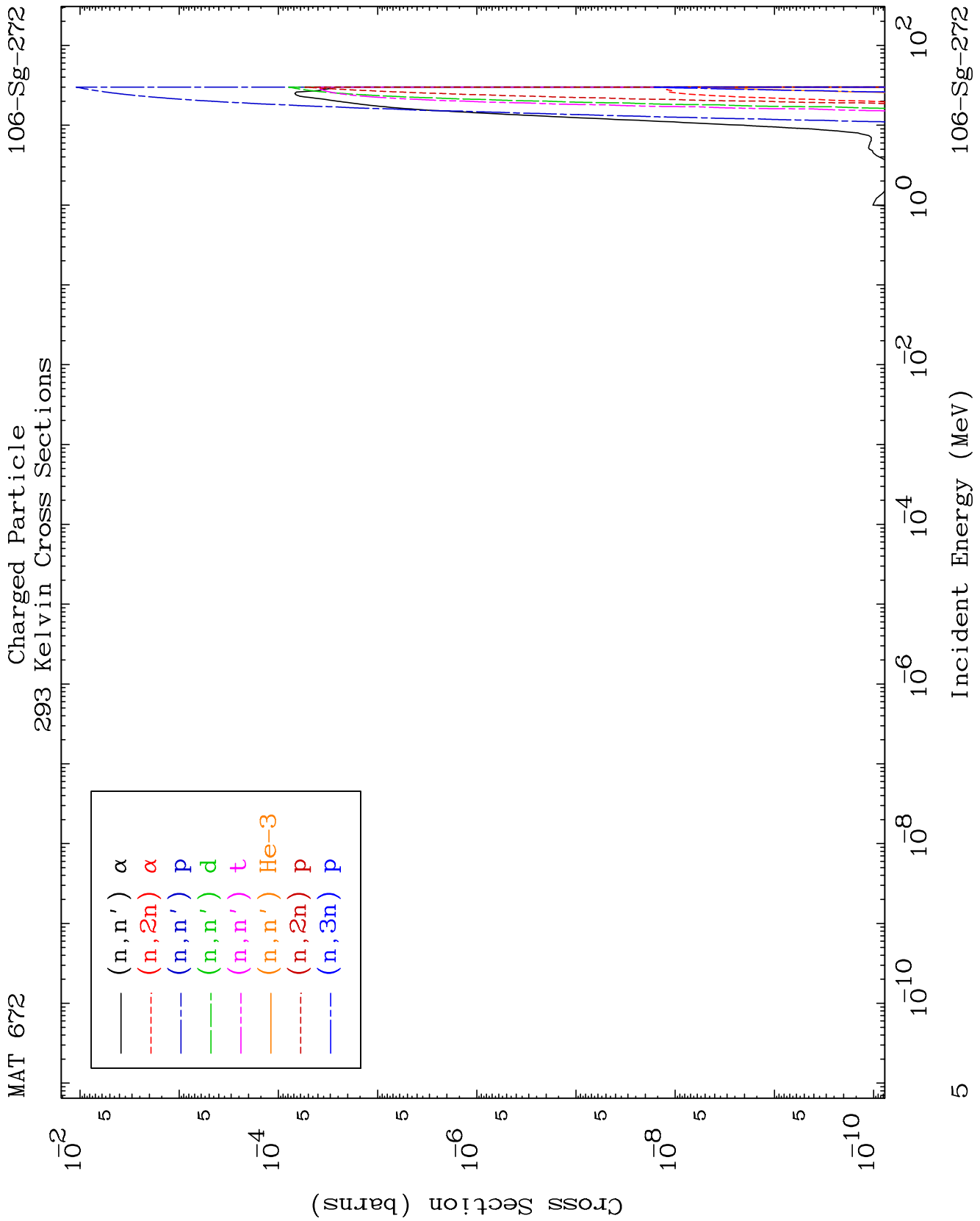


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Neutron Absorption  
293 Kelvin Cross Sections

106-Sg-272

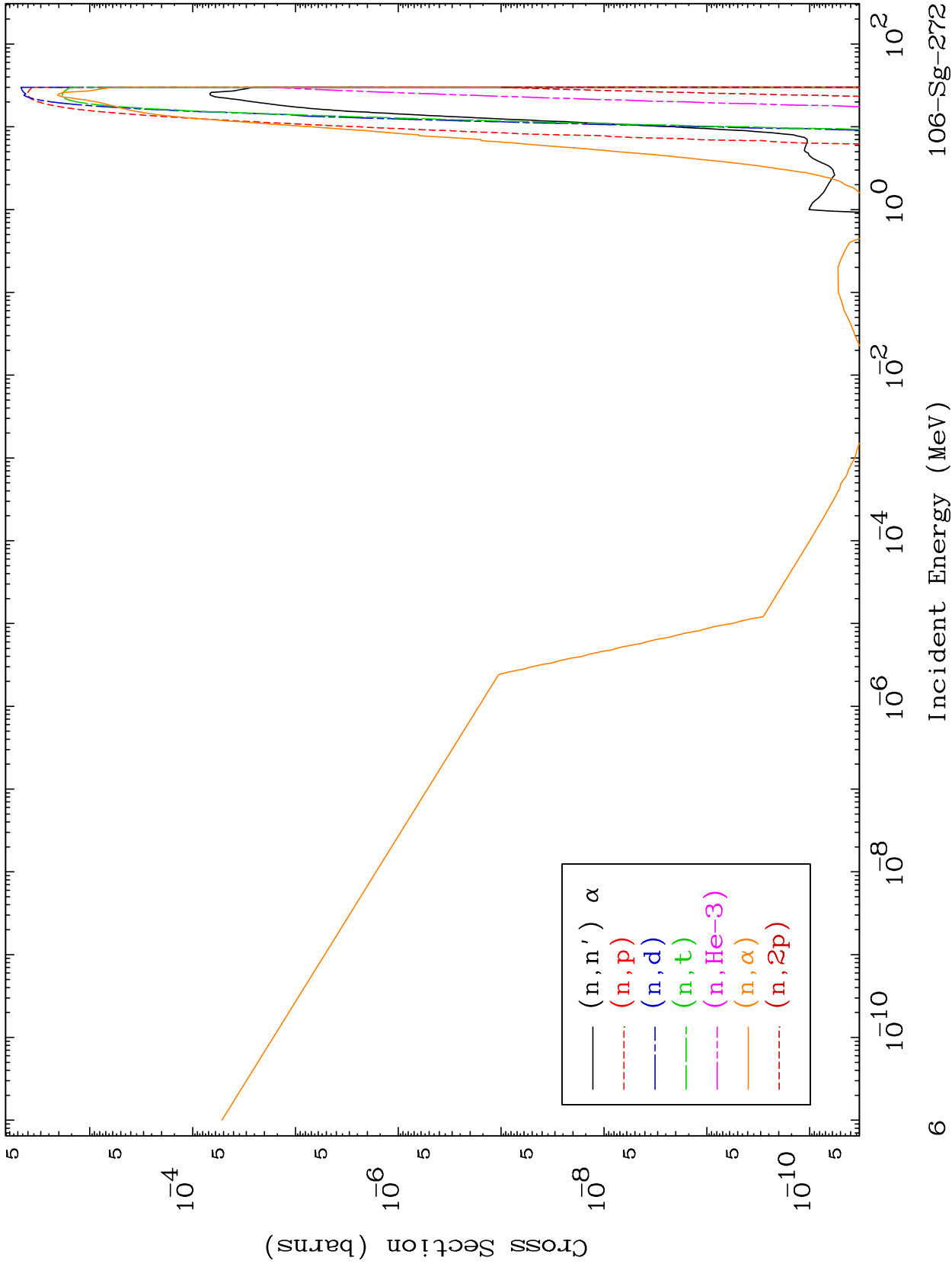




MAT 672

Charged Particle  
293 Kelvin Cross Sections

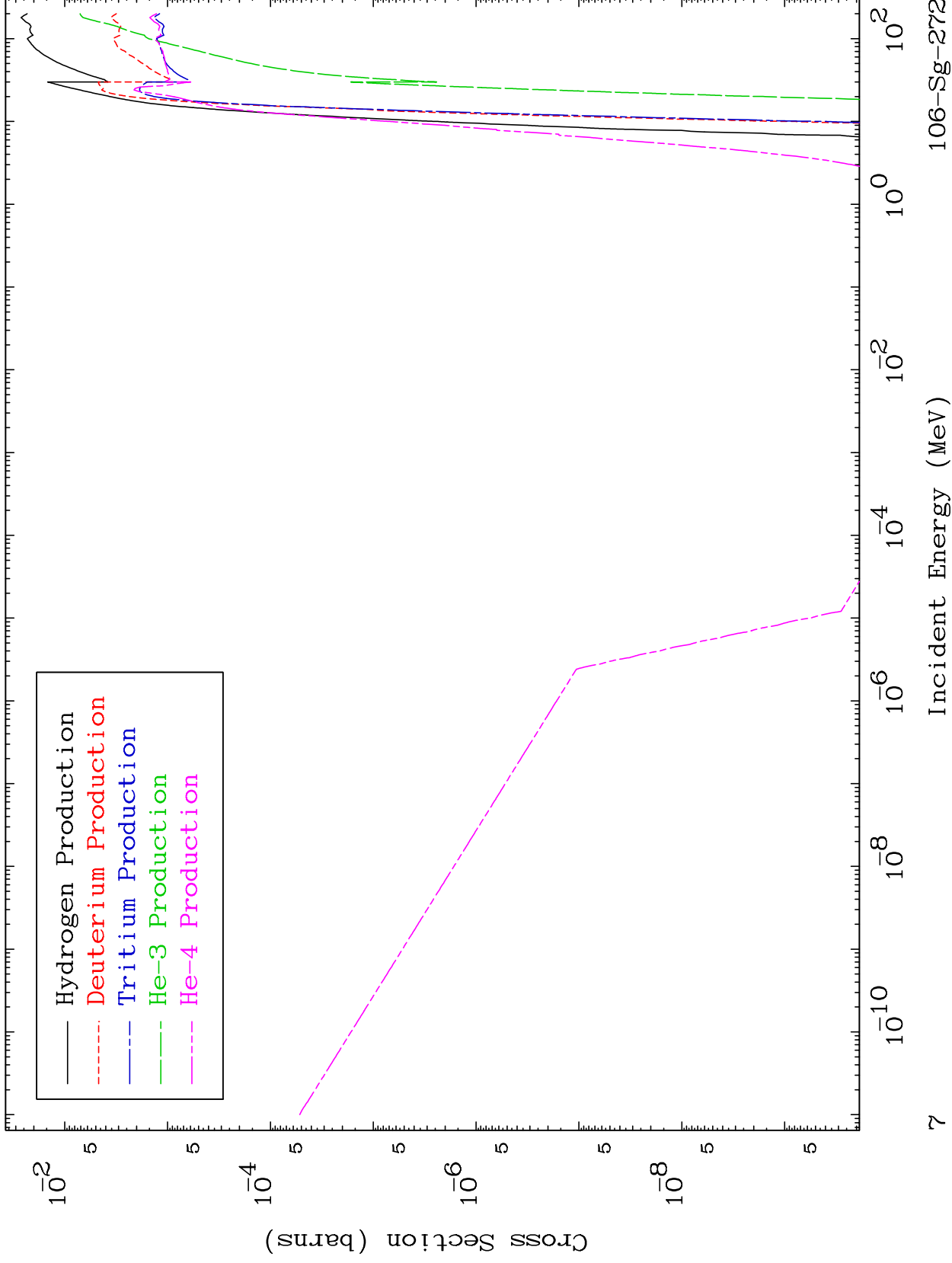
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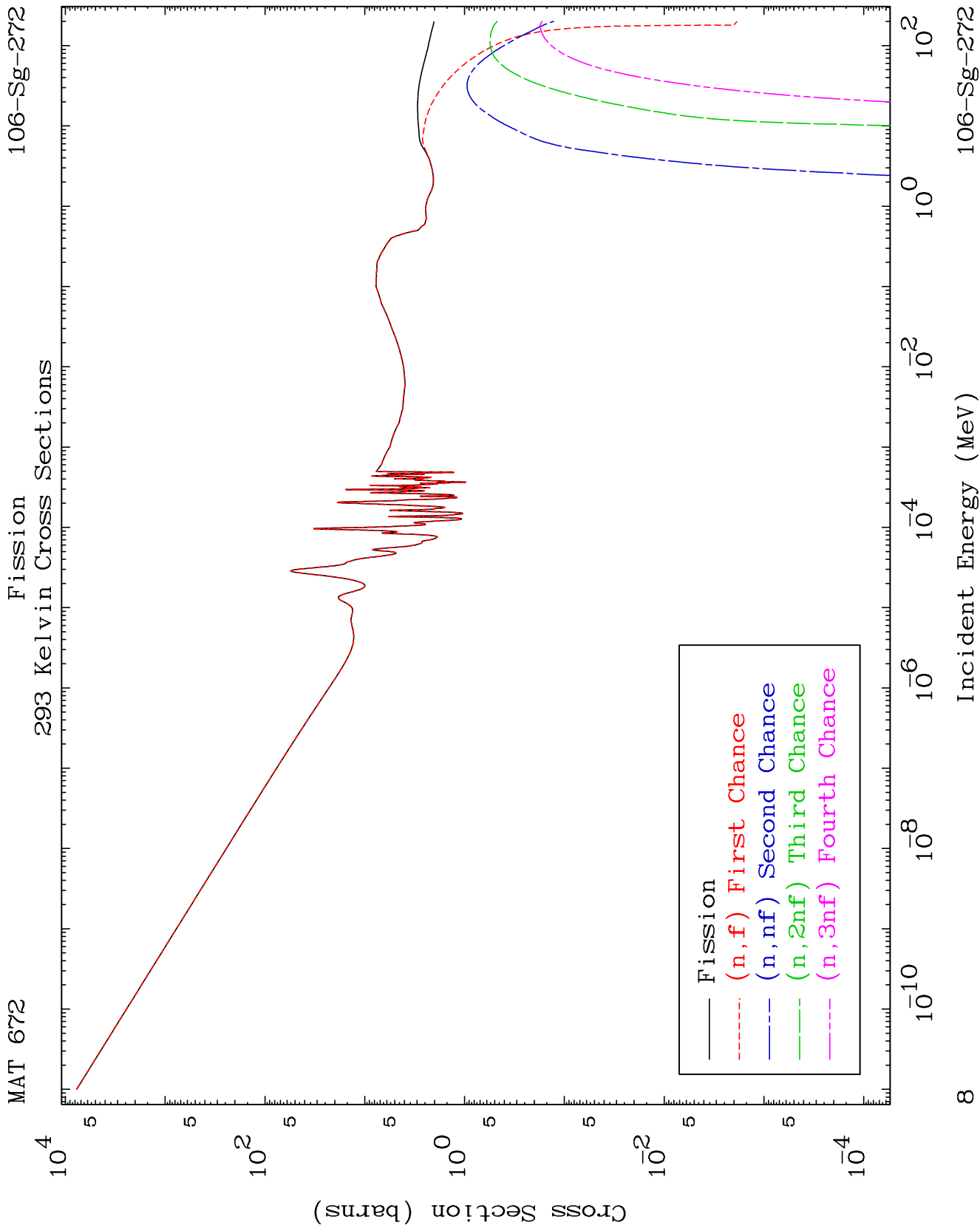
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Particle Production  
293 Kelvin Cross Sections

106-Sg-272



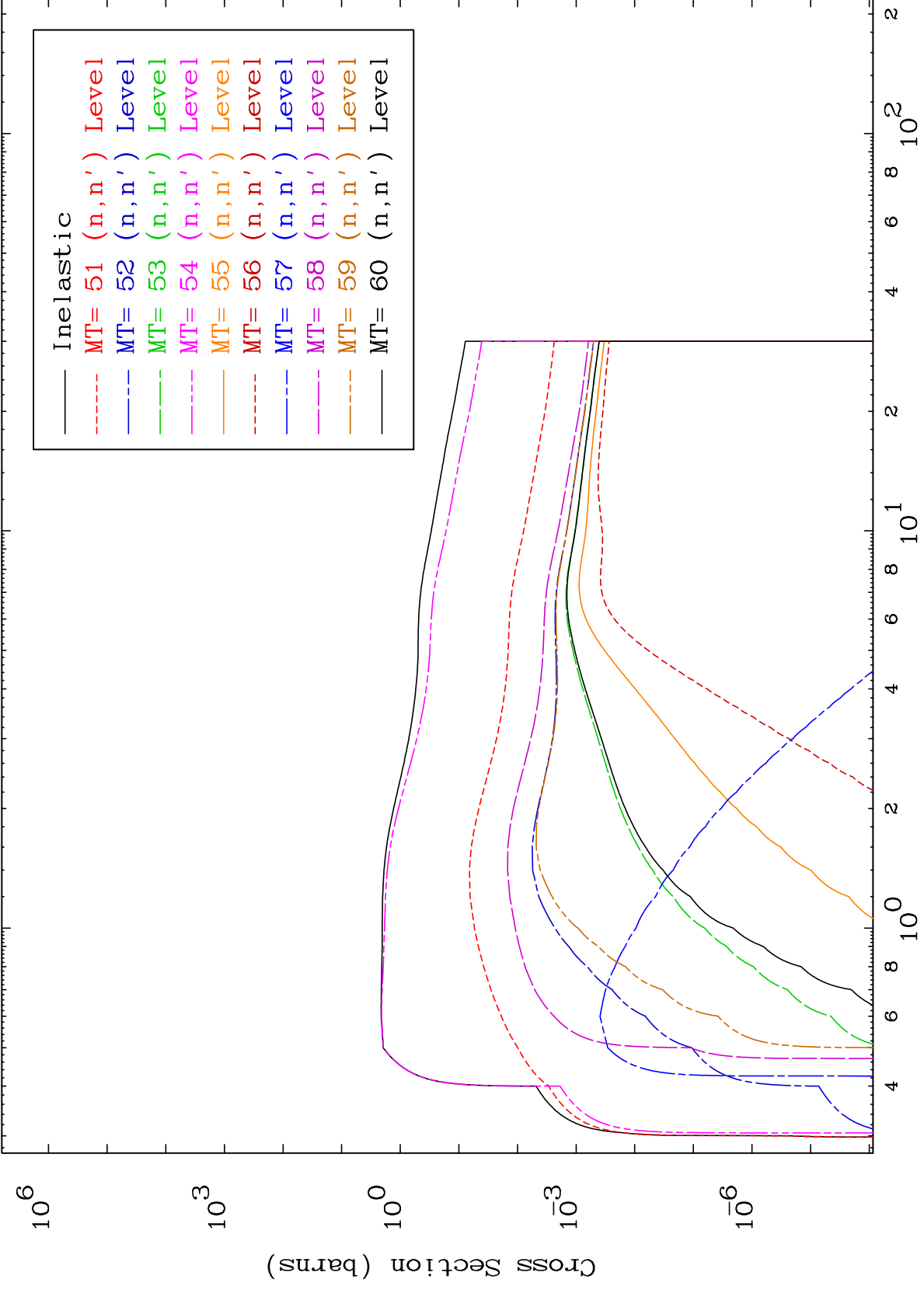




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(n,n') Levels  
293 Kelvin Cross Sections

106-Sg-272



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Incident Energy (MeV)

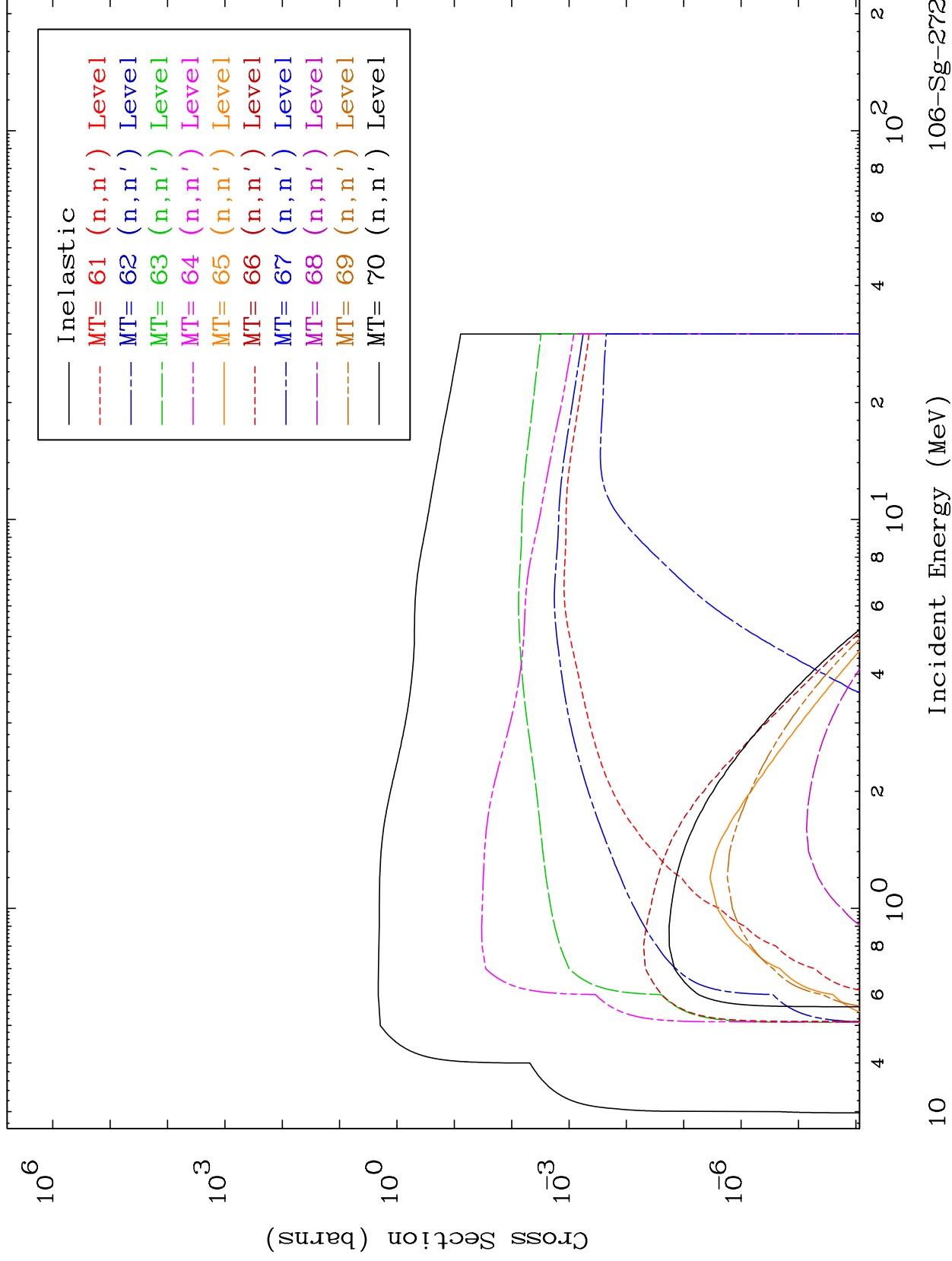
106-Sg-272

MAT 672

(n,n') Levels

106-Sg-272

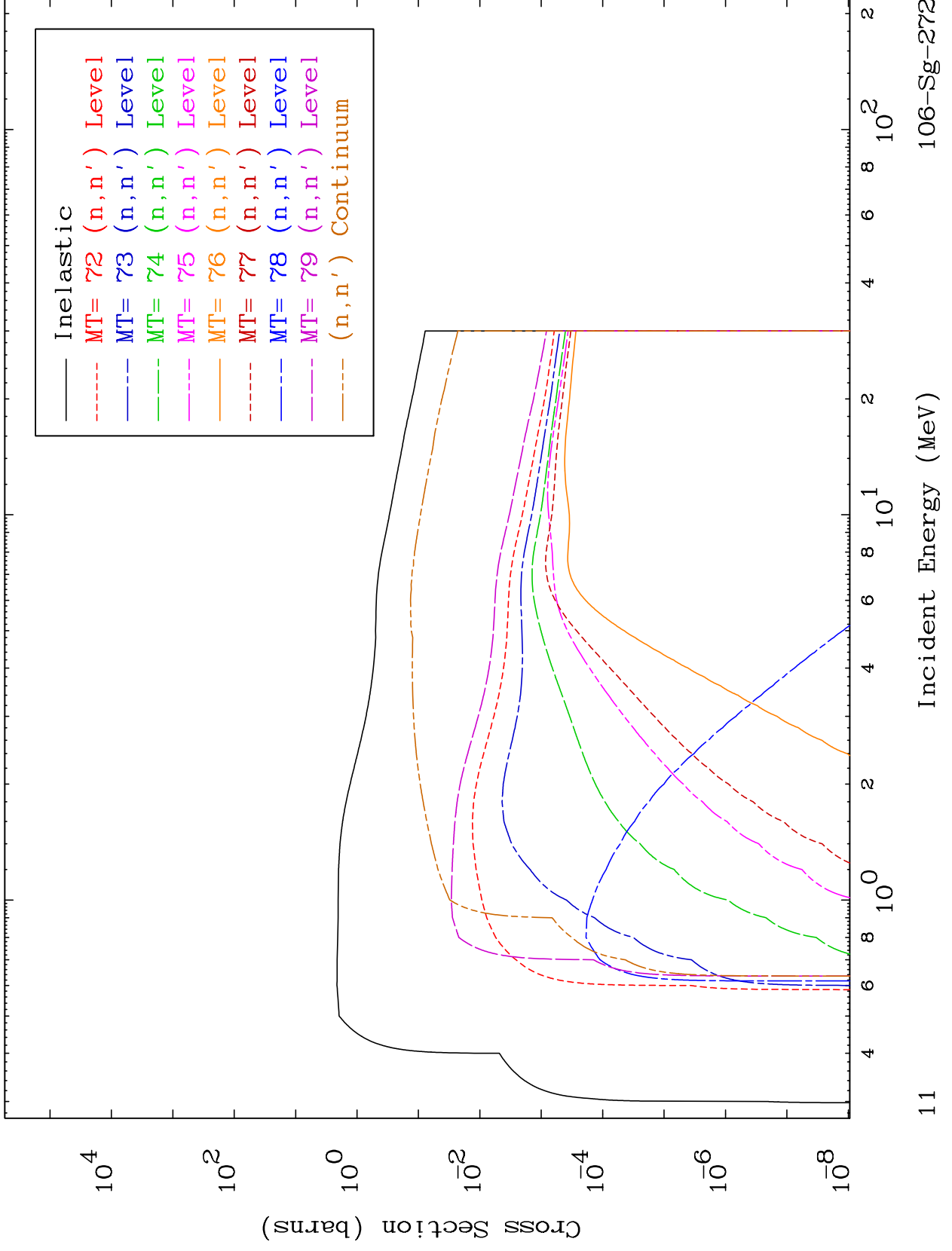
293 Kelvin Cross Sections



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Incident Energy (MeV)

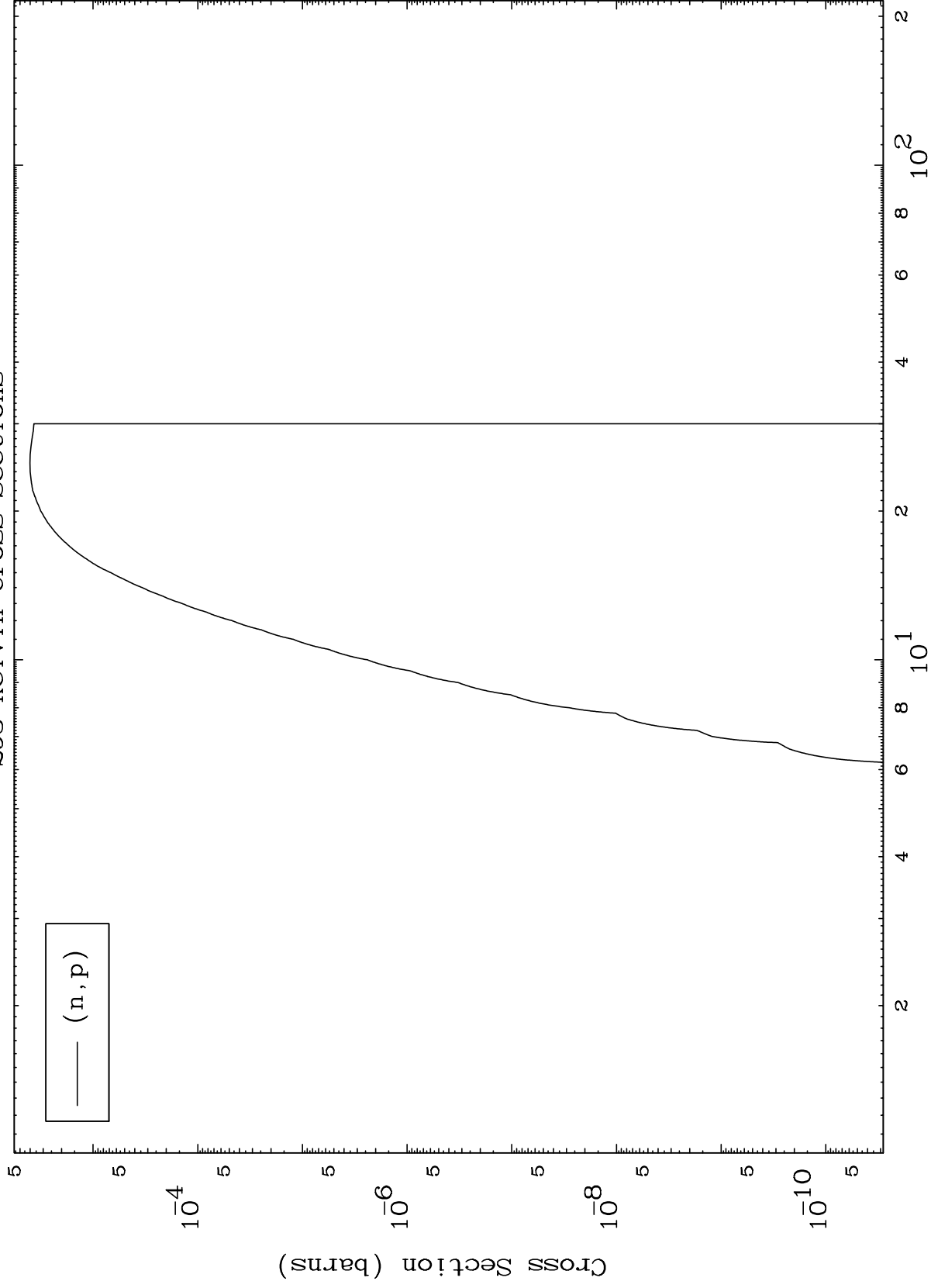
106-Sg-272



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(n,p) Levels  
293 Kelvin Cross Sections

106-Sg-272



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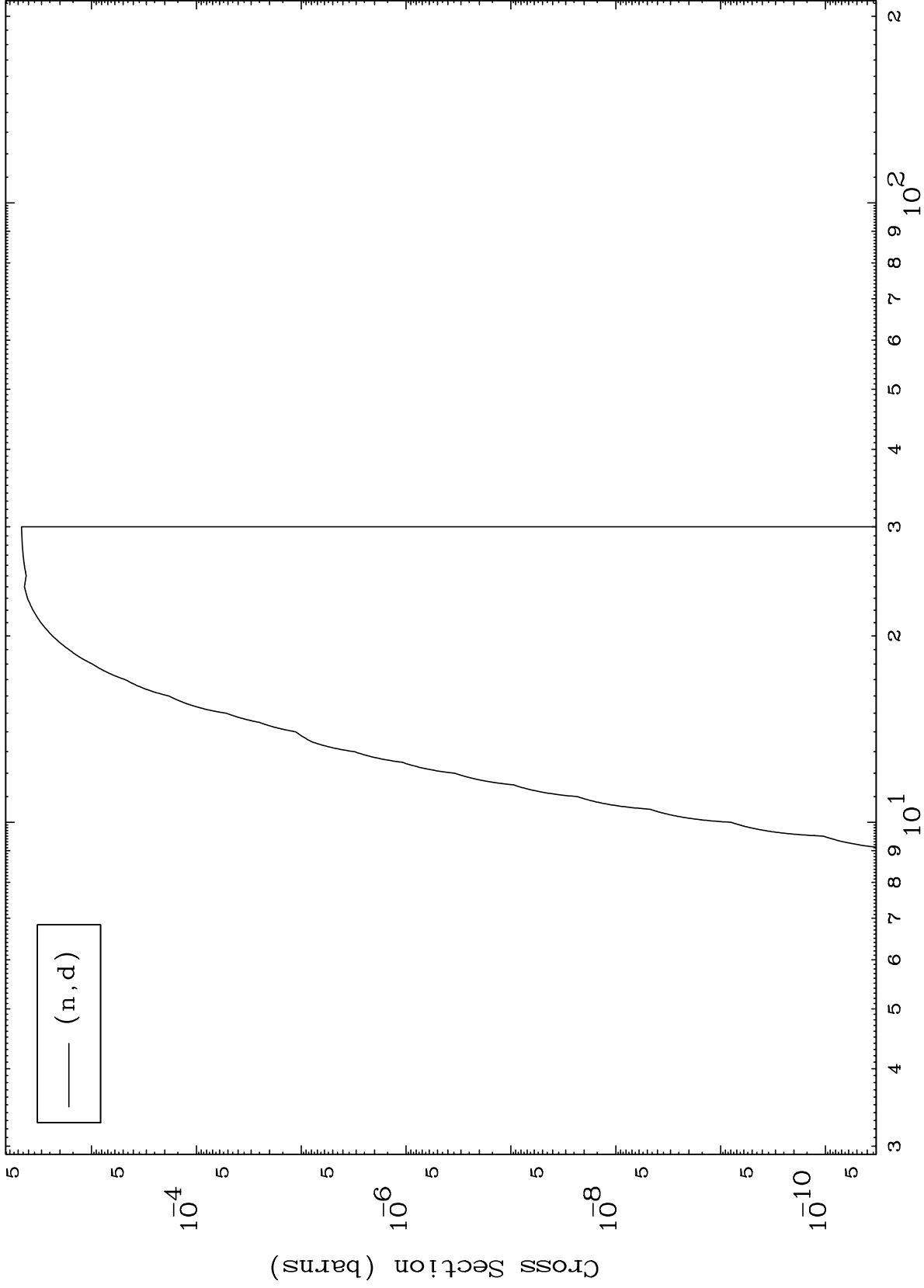
Incident Energy (MeV)

106-Sg-272

MAT 672

(n,d) Levels  
293 Kelvin Cross Sections

106-Sg-272



13

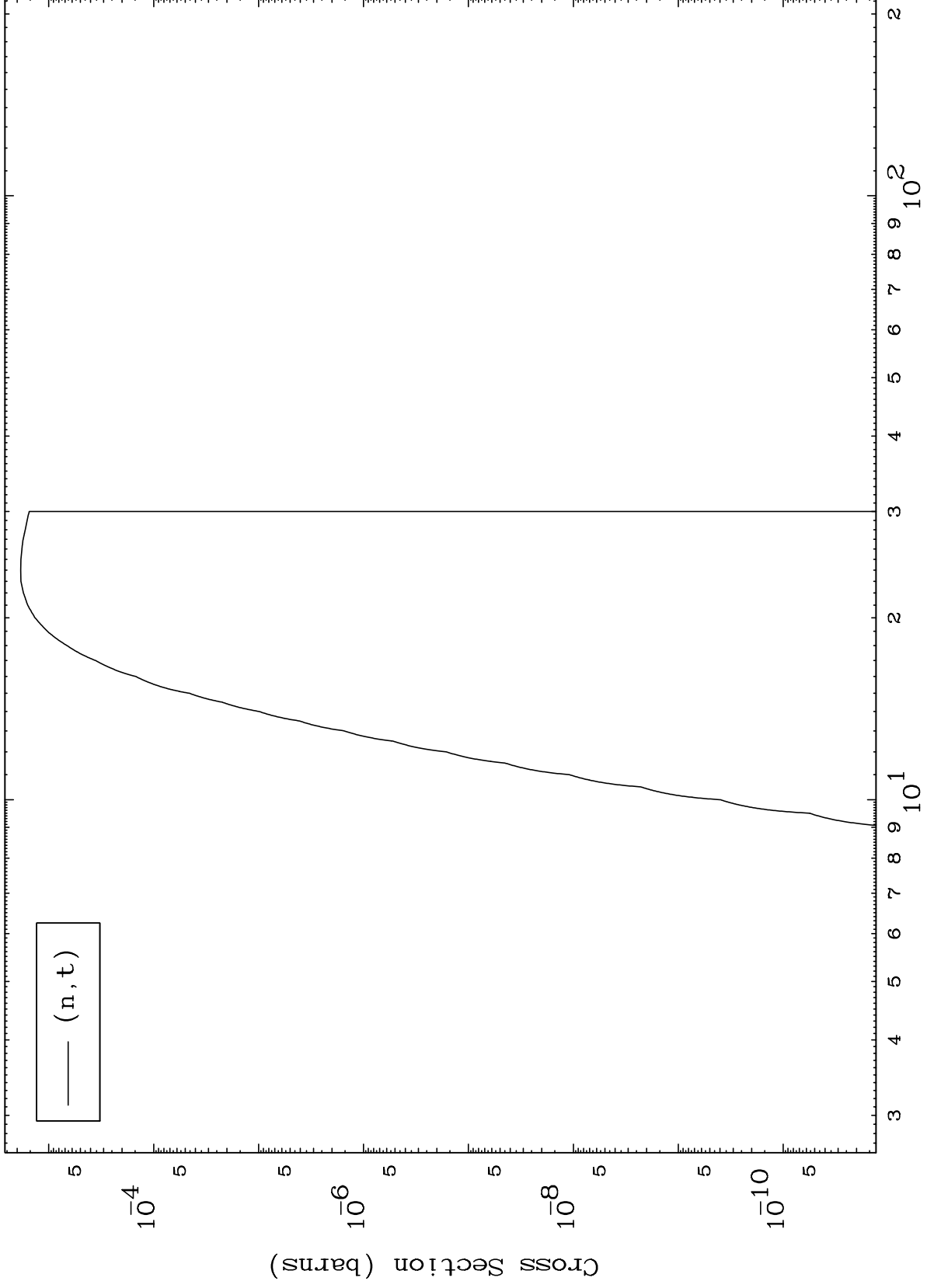
Incident Energy (MeV)

106-Sg-272

MAT 672

(n,t) Levels  
293 Kelvin Cross Sections

106-Sg-272



14

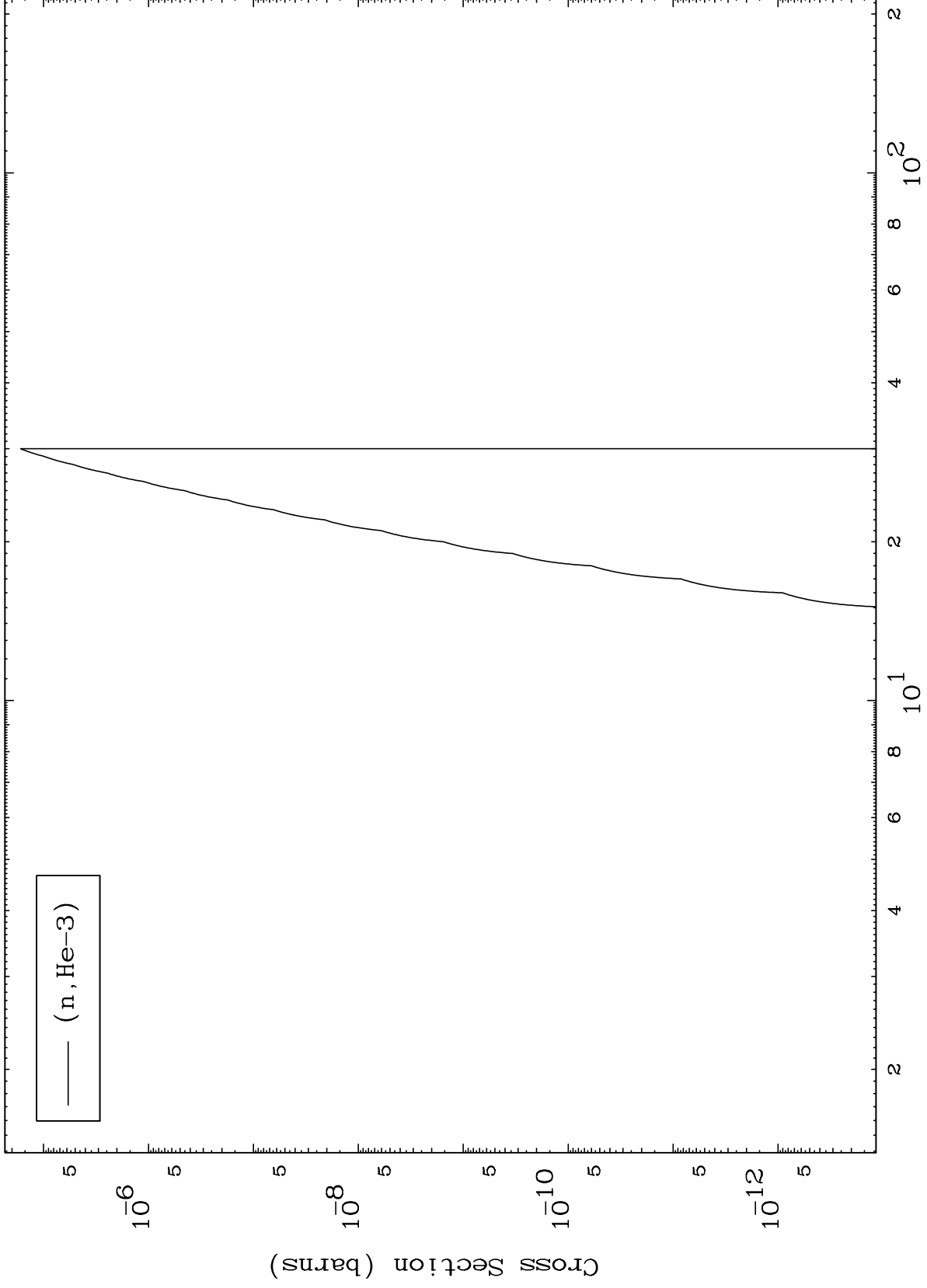
Incident Energy (MeV)

106-Sg-272

MAT 672

(n,He3) Levels  
293 Kelvin Cross Sections

106-Sg-272



15

Incident Energy (MeV)

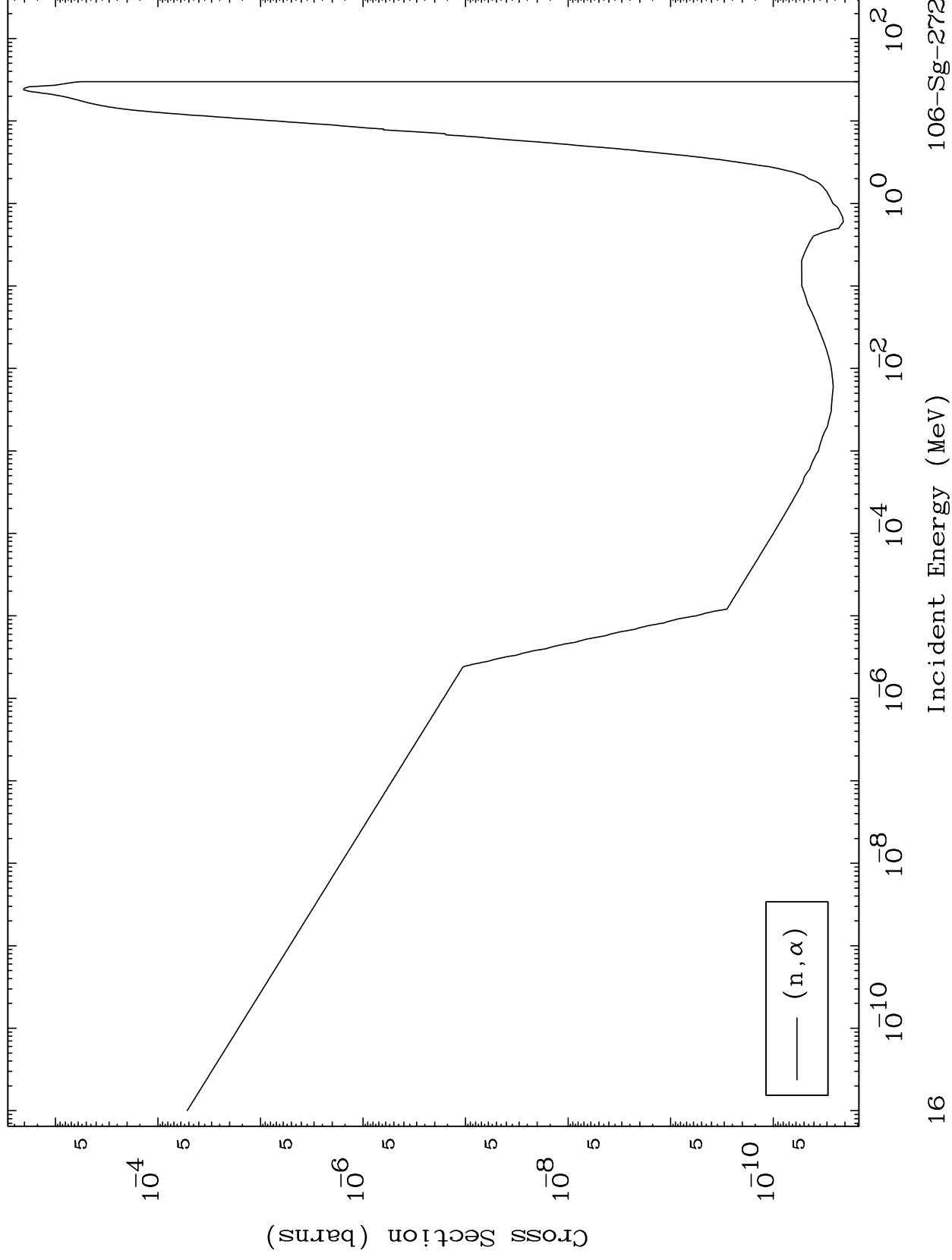
106-Sg-272



MAT 672

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

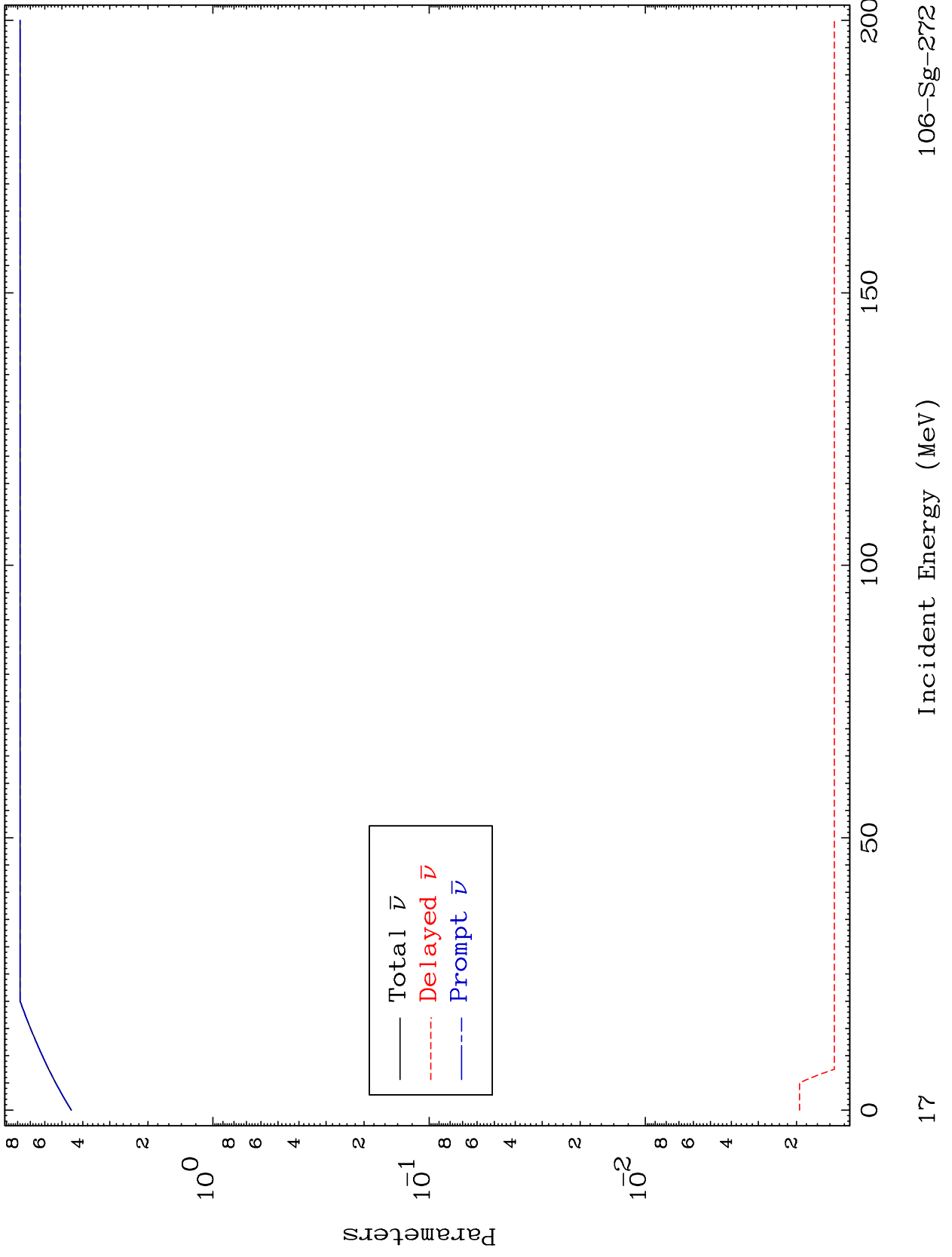
106-Sg-272



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Energy Release  
Parameters

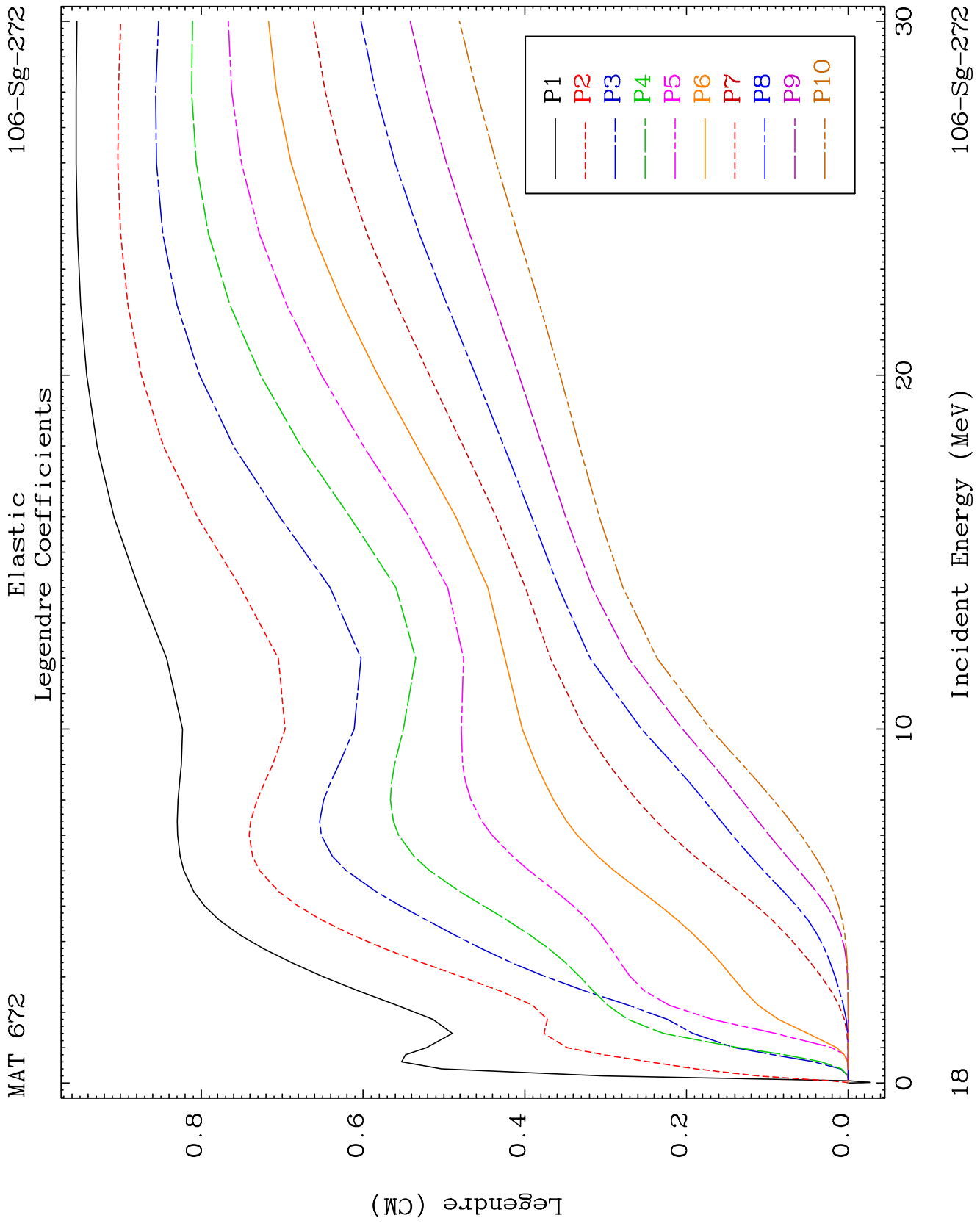
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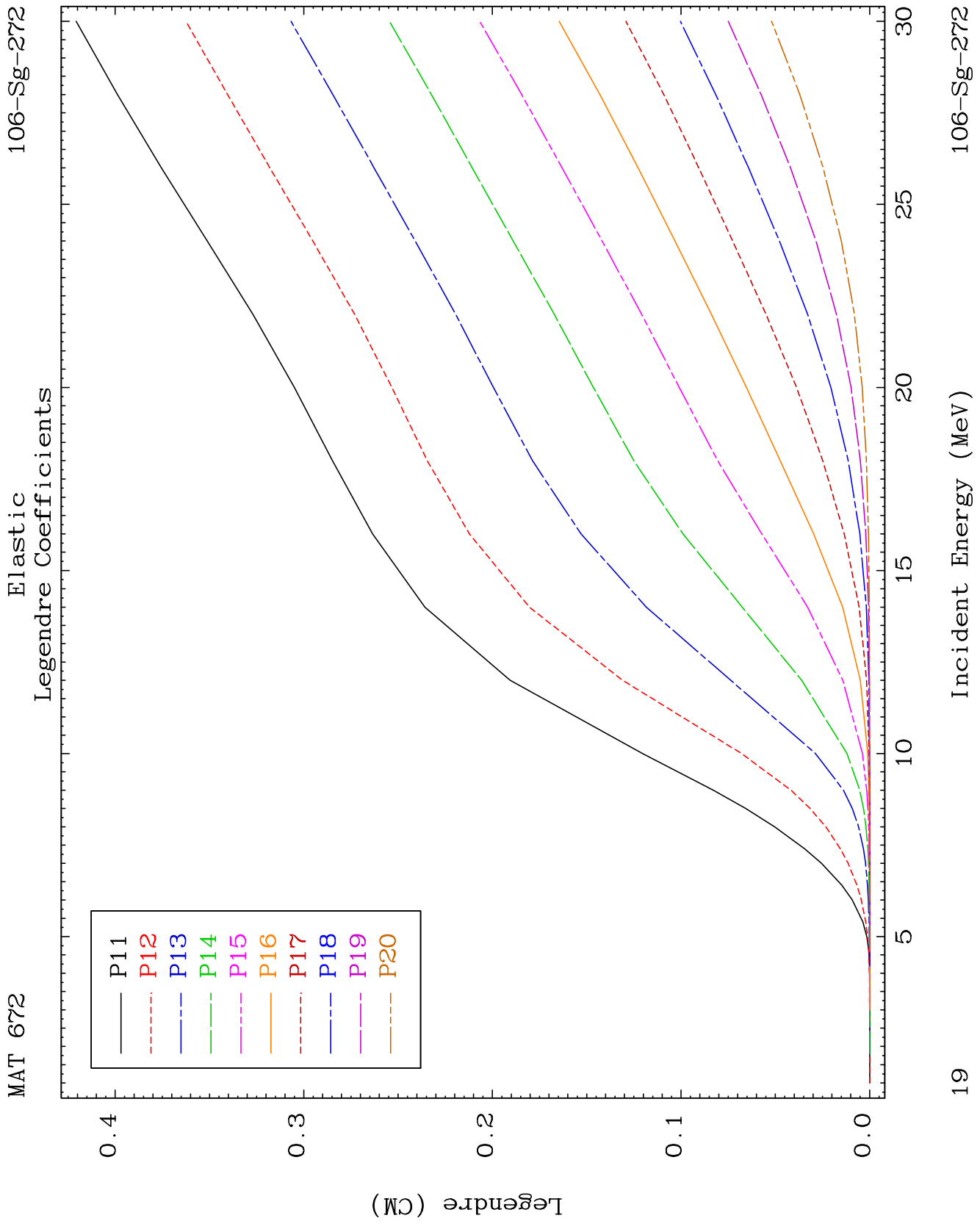


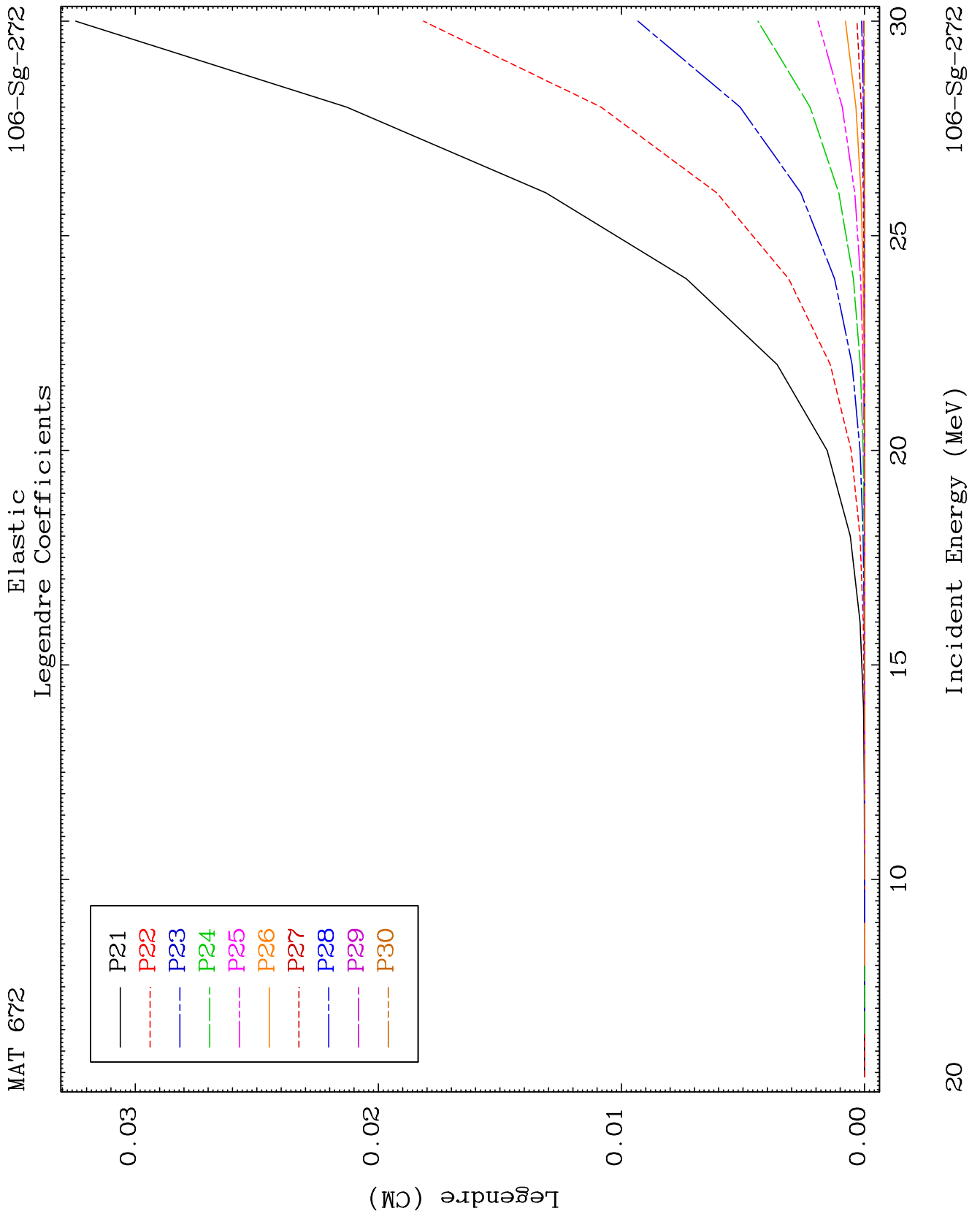
17

Incident Energy (MeV)

106-Sg-272





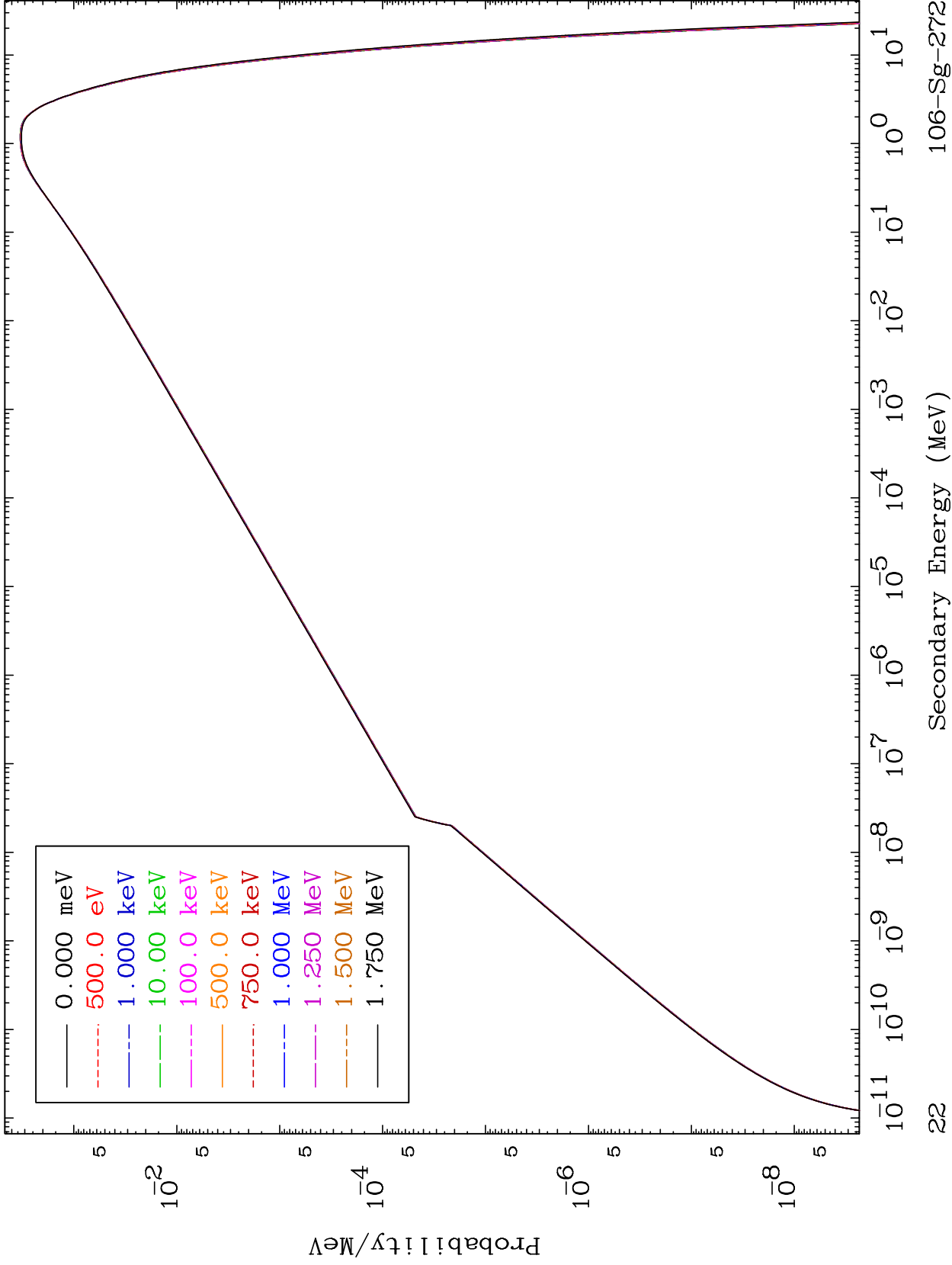




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### Fission Energy Distributions

106-Sg-272



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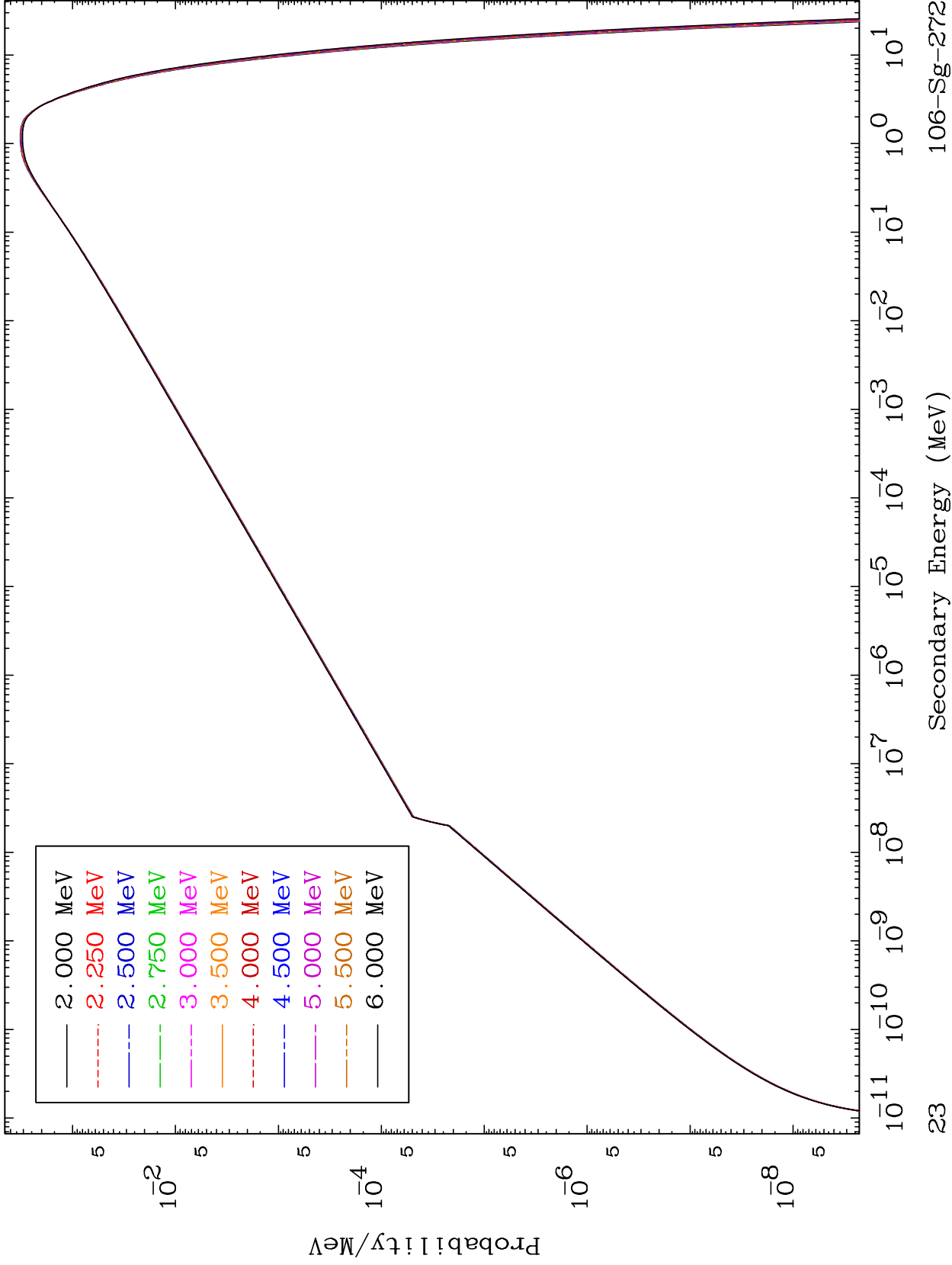
Secondary Energy (MeV)

106-Sg-272

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### Fission Energy Distributions

106-Sg-272

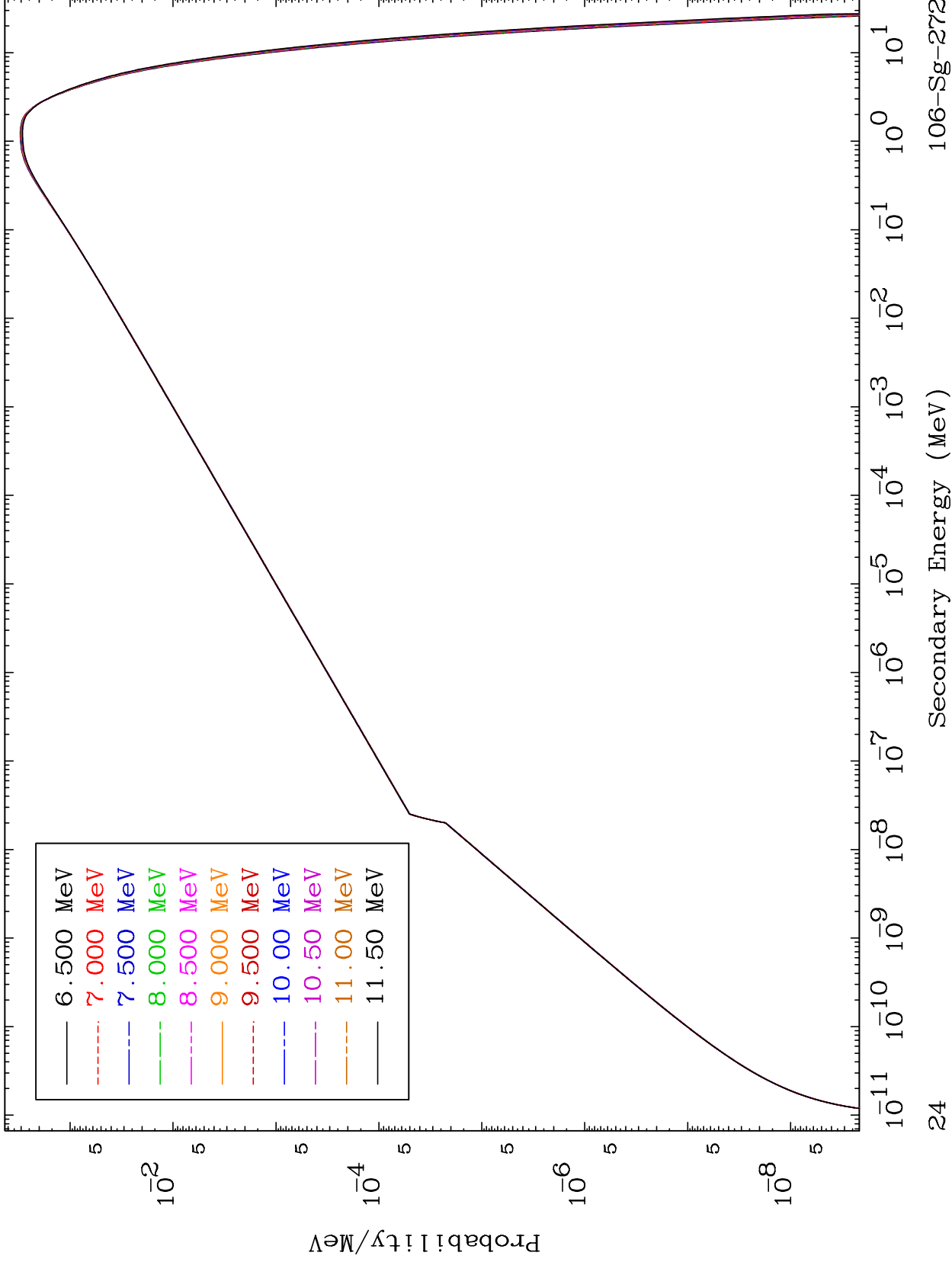




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### Fission Energy Distributions

106-Sg-272



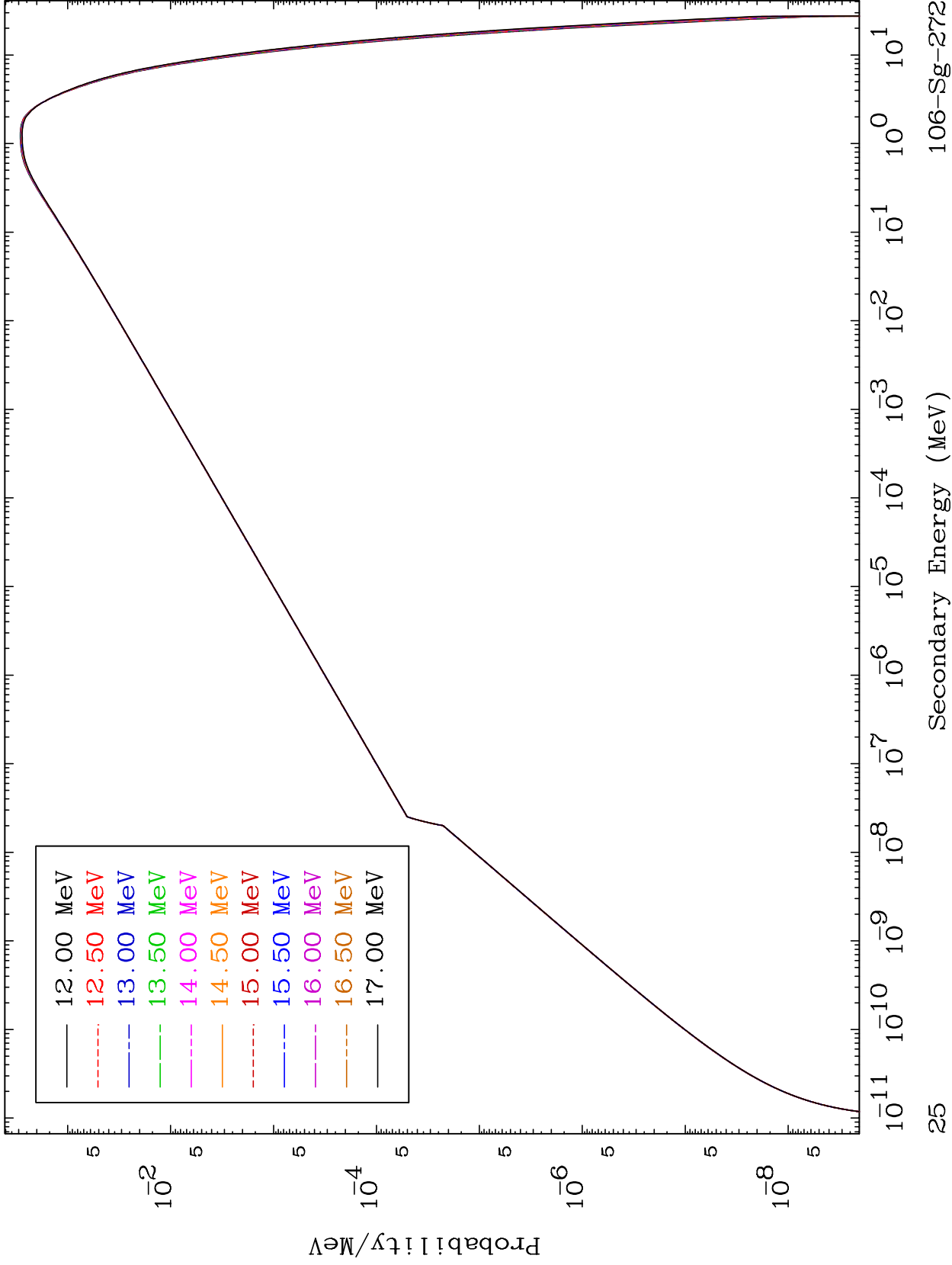
24

106-Sg-272

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### Fission Energy Distributions

106-Sg-272



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106-Sg-272

MAT 672

### Fission Energy Distributions

106-Sg-272

