

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

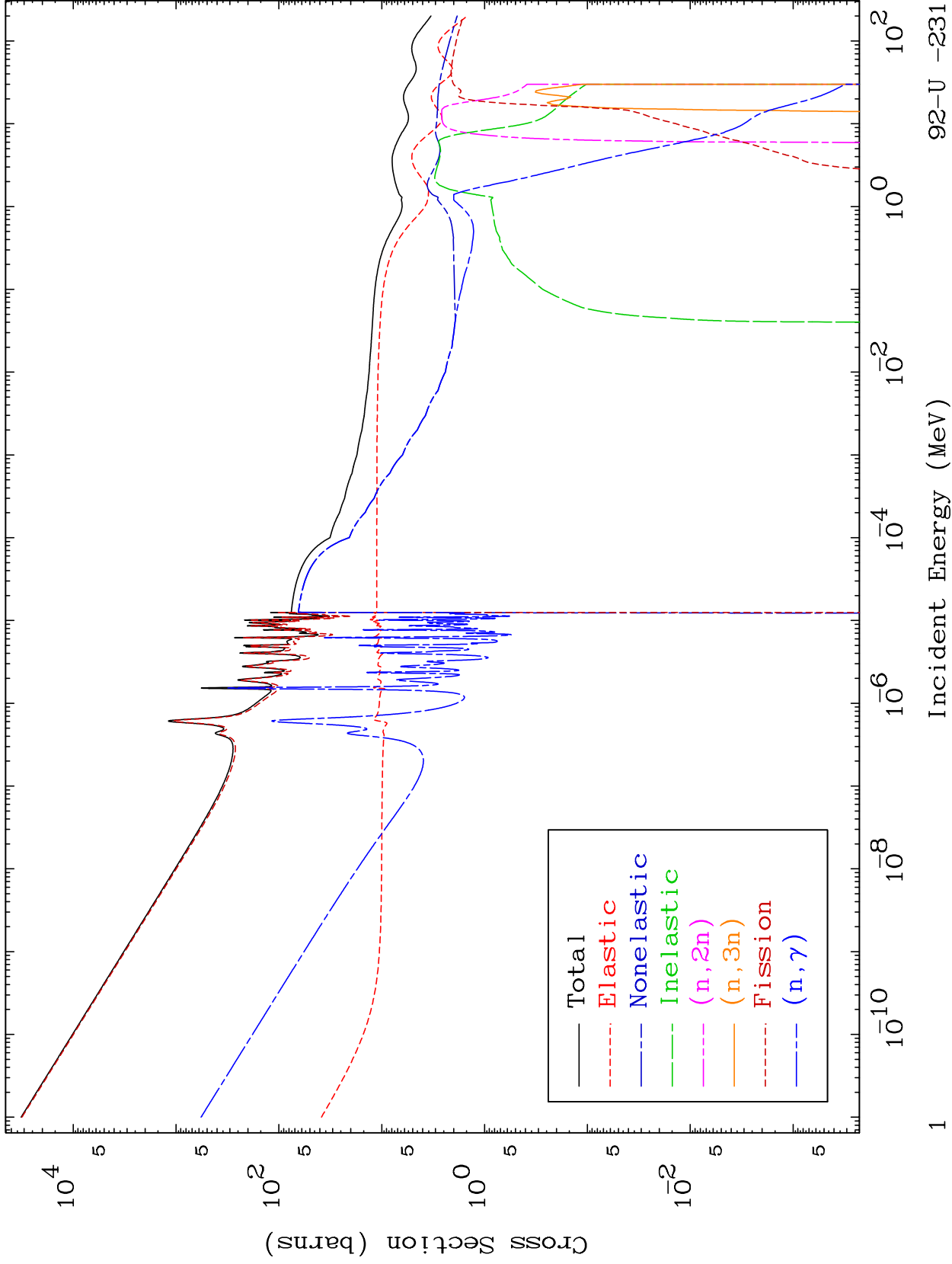
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9216

Neutron Major
293 Kelvin Cross Sections

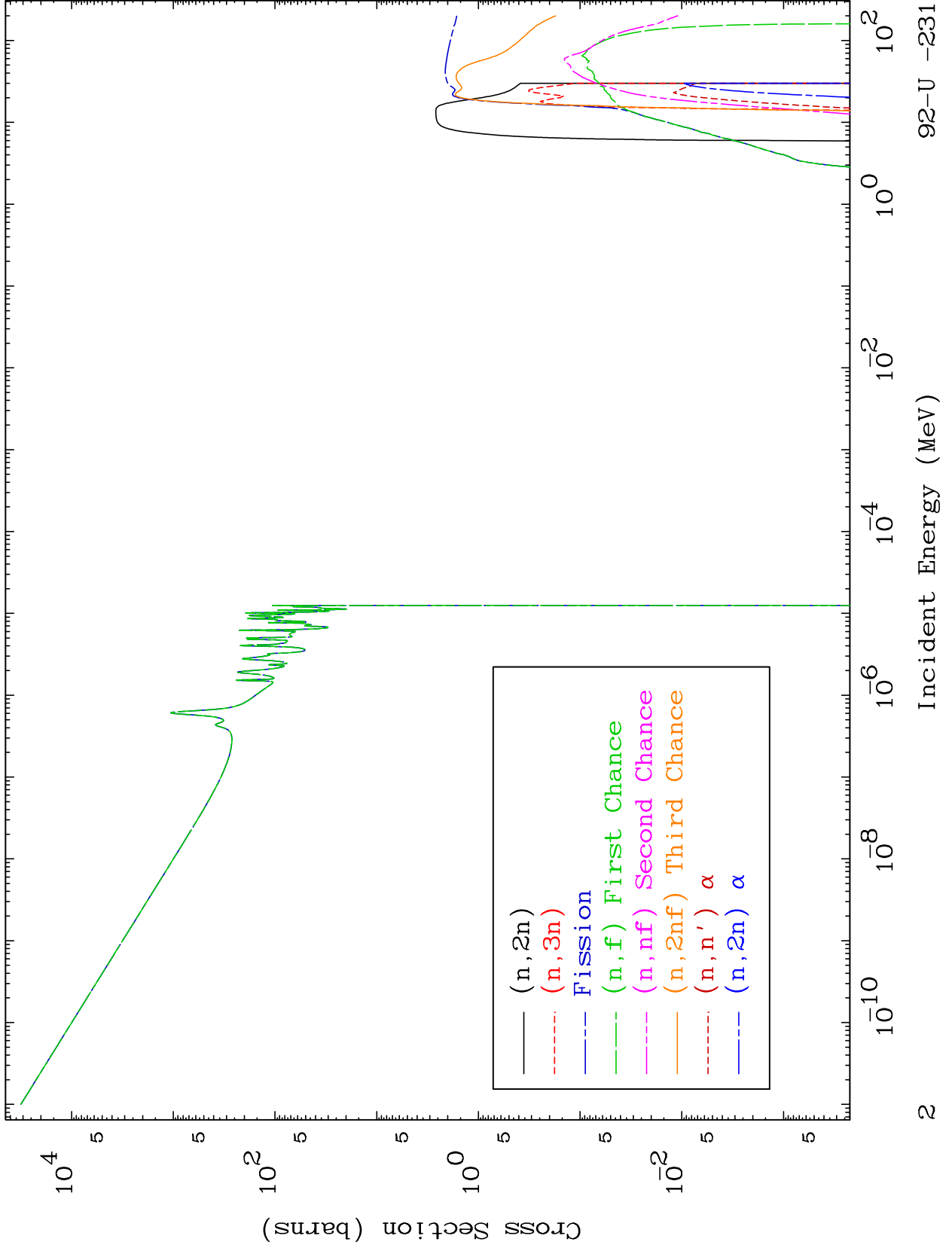
92-U -231

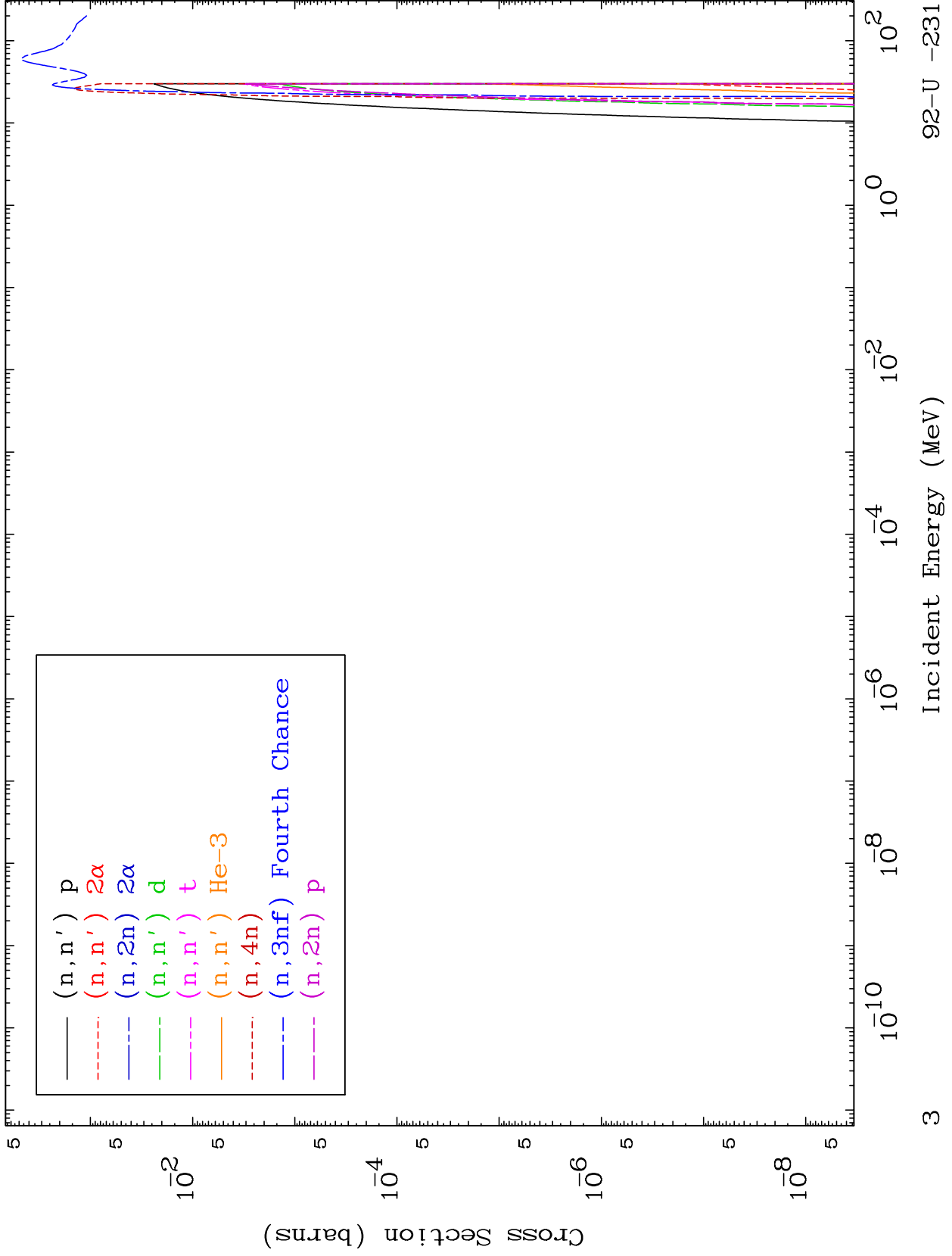


MAT 9216

Neutron Absorption
293 Kelvin Cross Sections

92-U -231

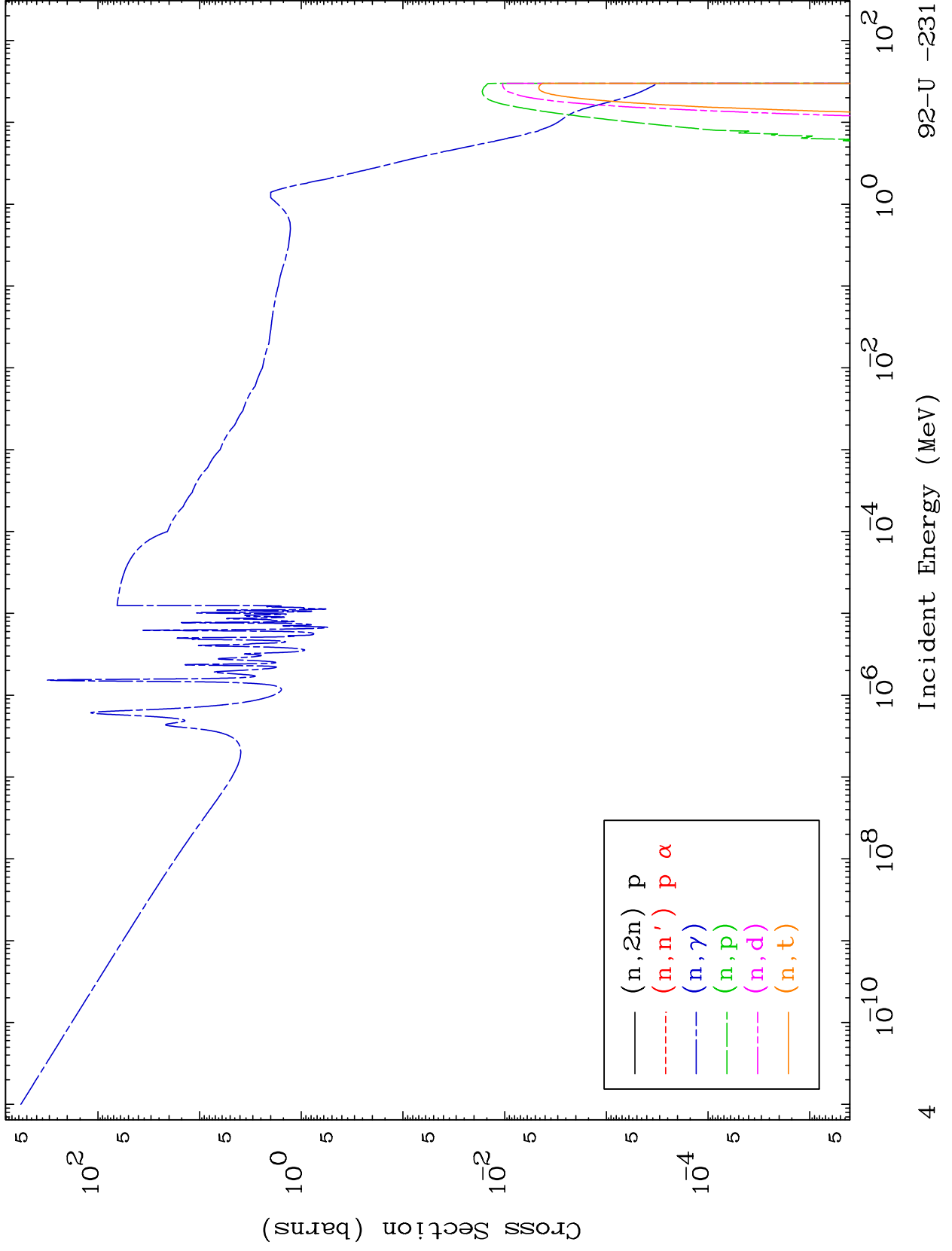


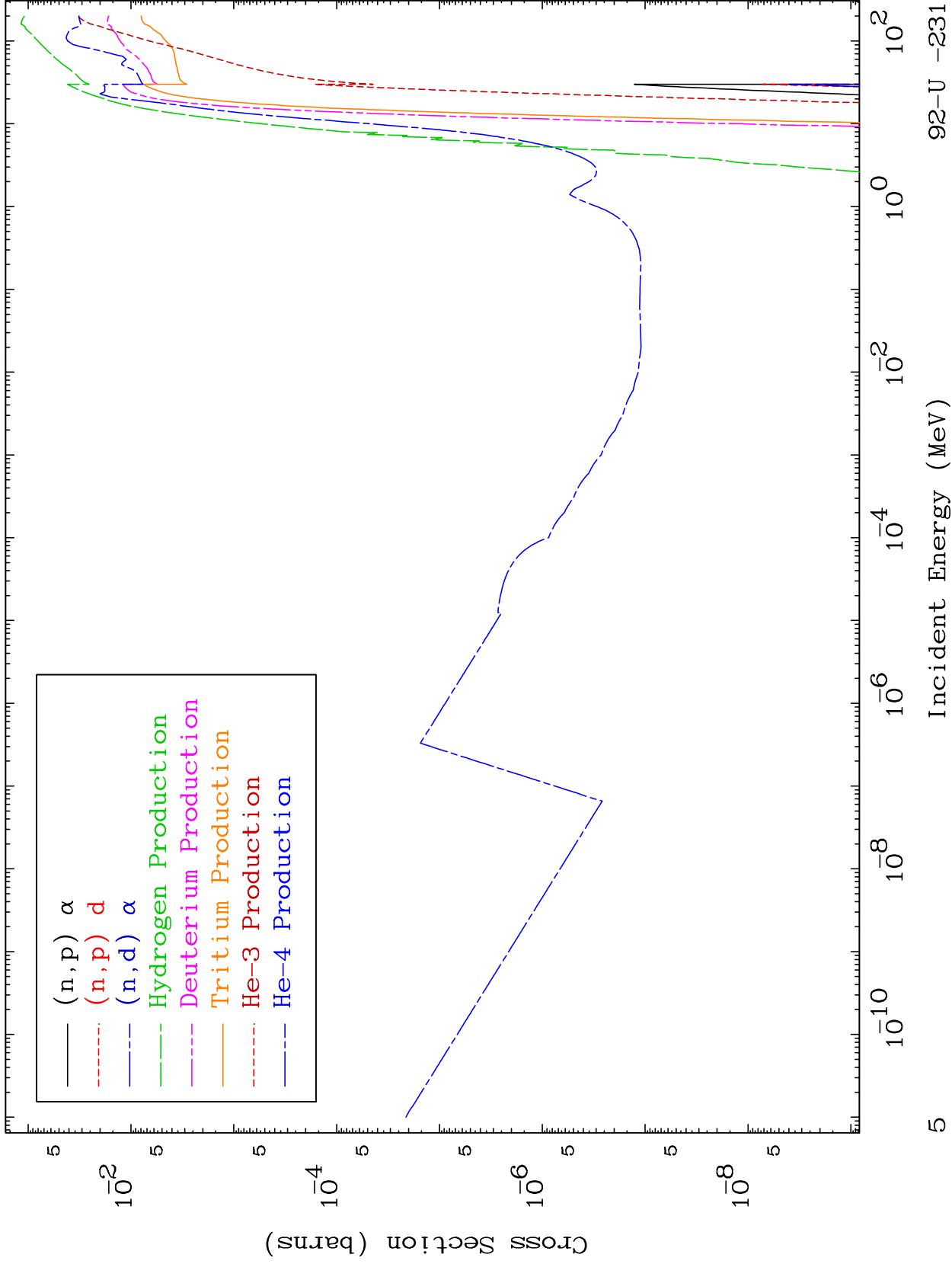


MAT 9216

Neutron Absorption
293 Kelvin Cross Sections

92-U -231

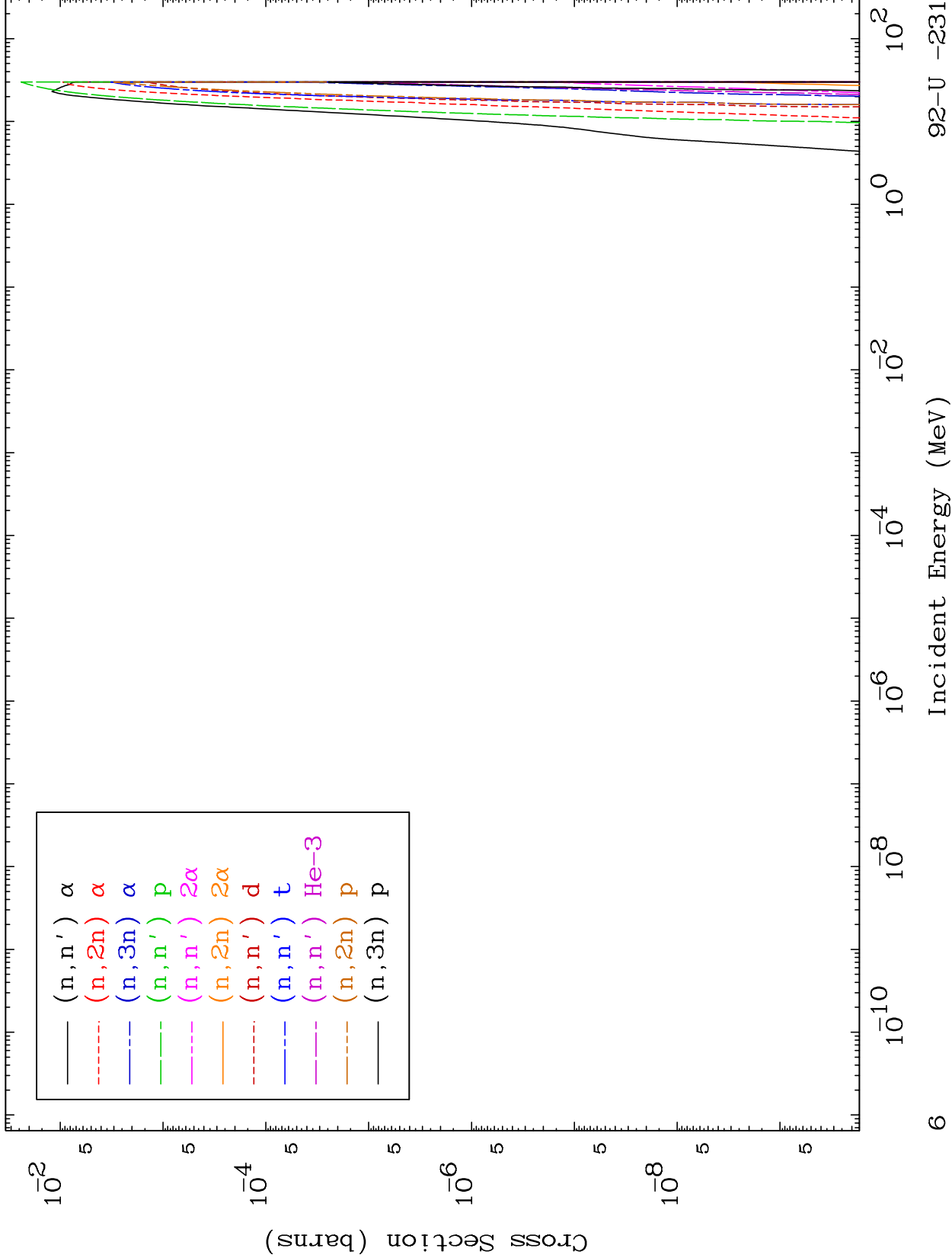




MAT 9216

Charged Particle
293 Kelvin Cross Sections

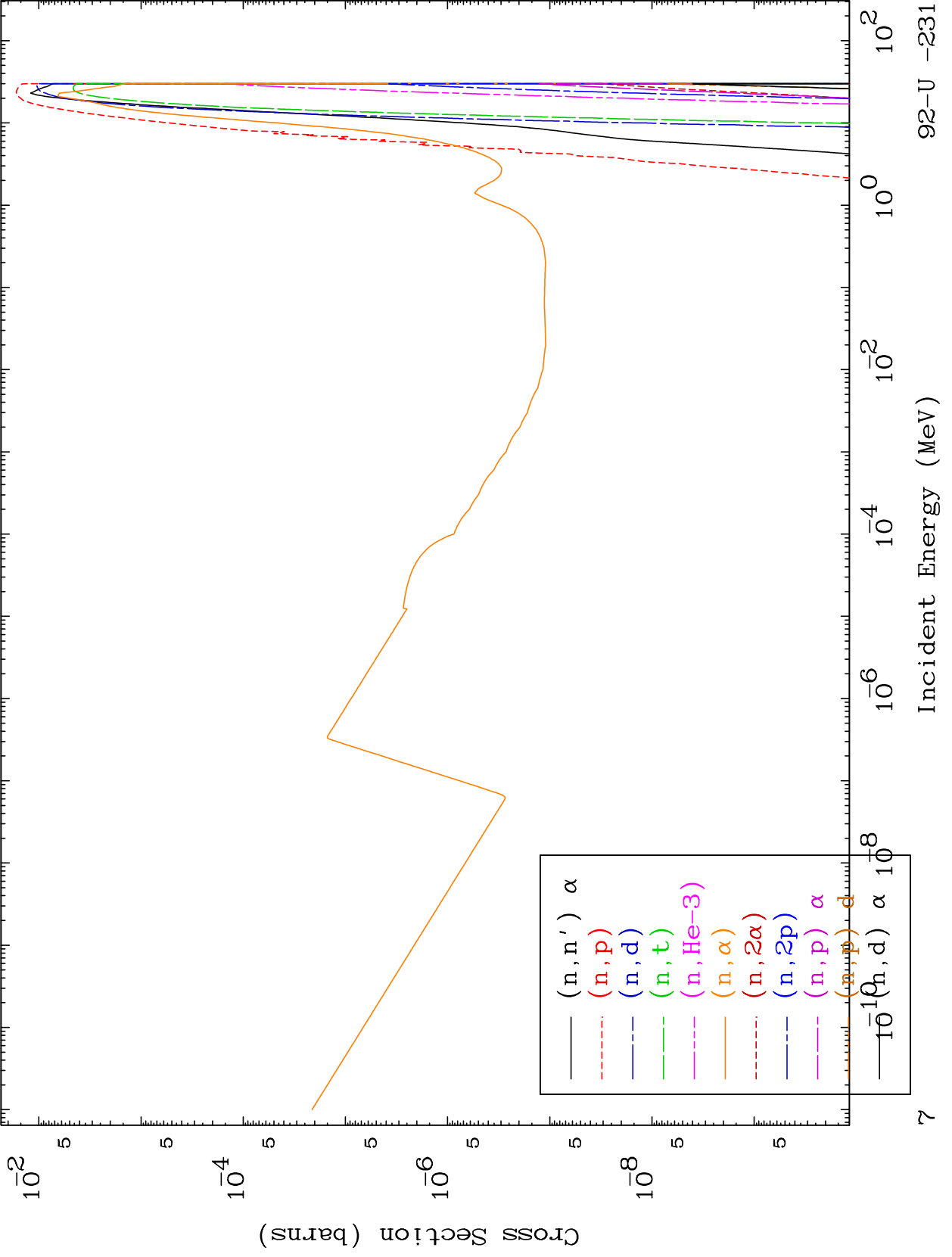
92-U -231

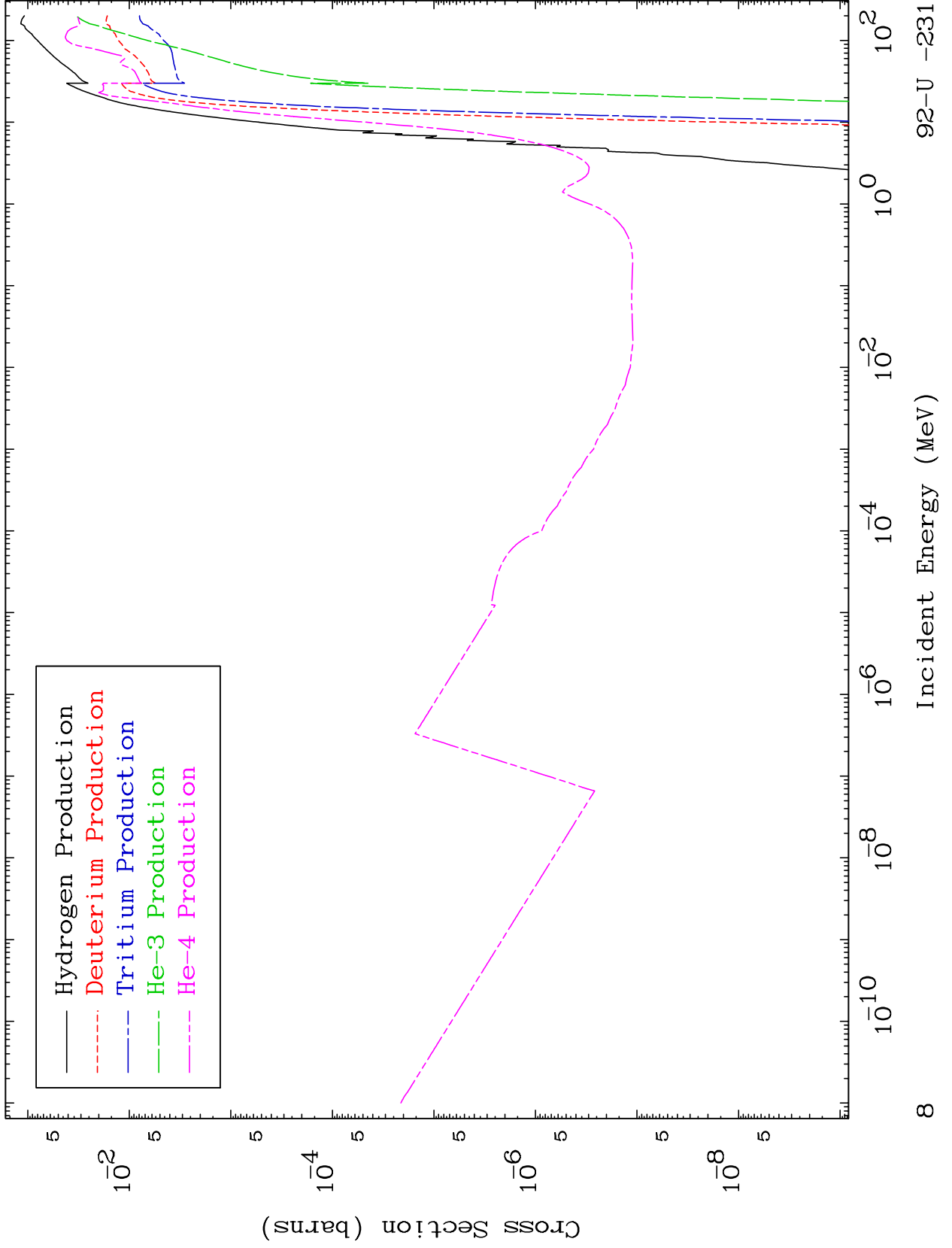


MAT 9216

Charged Particle
293 Kelvin Cross Sections

92-U -231

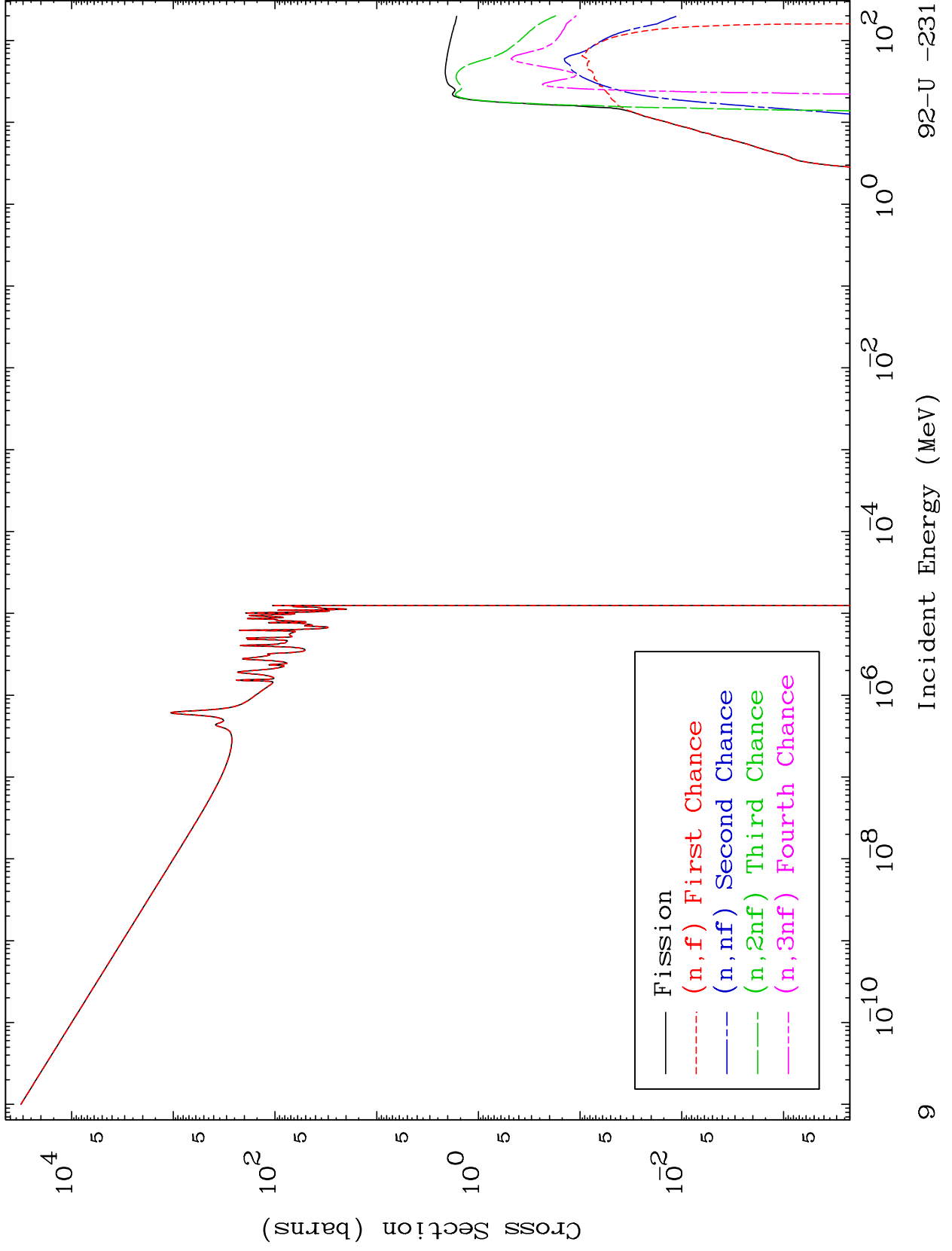




MAT 9216

Fission
293 Kelvin Cross Sections

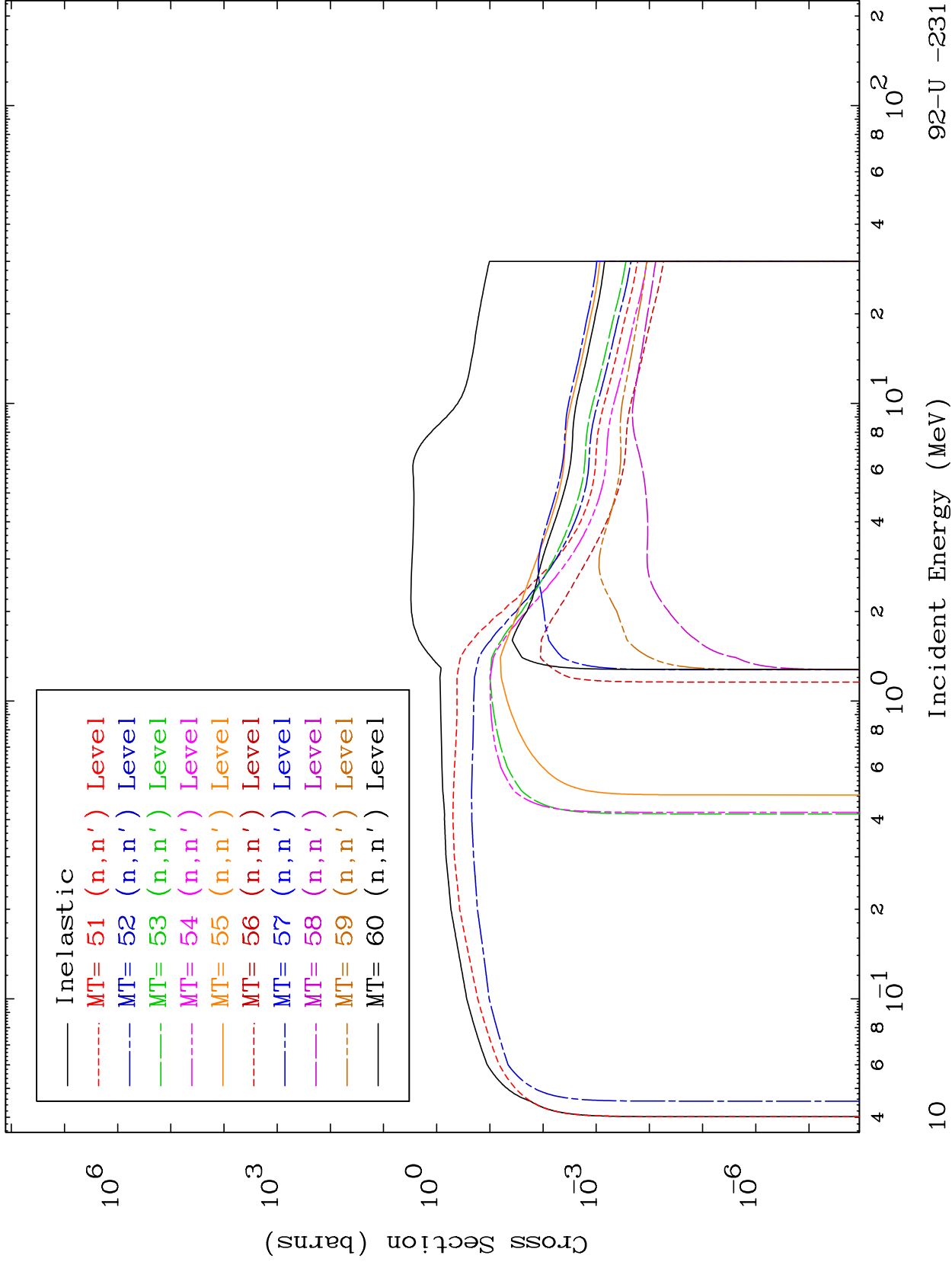
92-U -231



MAT 9216

(n,n') Levels
293 Kelvin Cross Sections

92-U -231



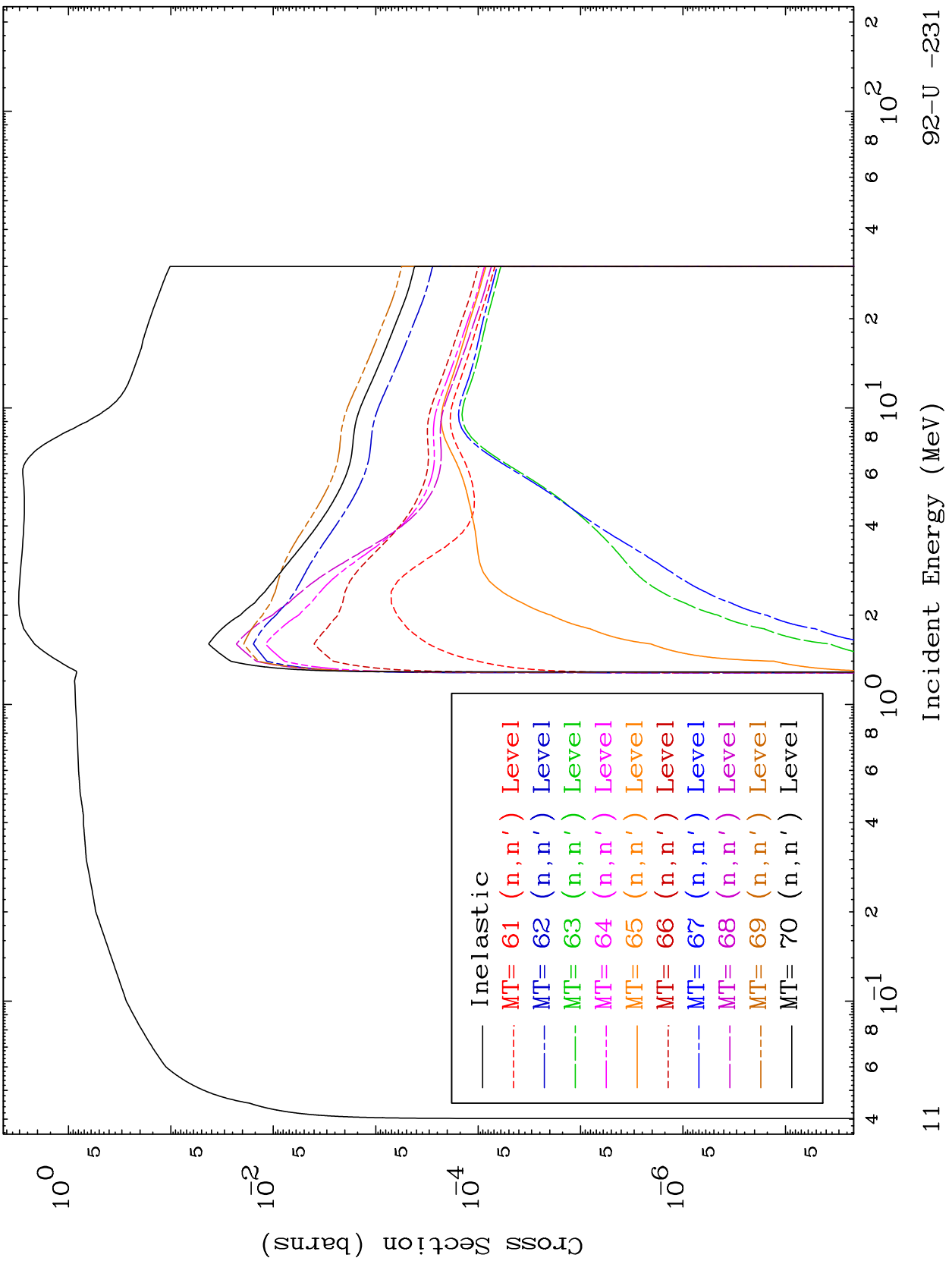
10

92-U -231

MAT 9216

92-U -231

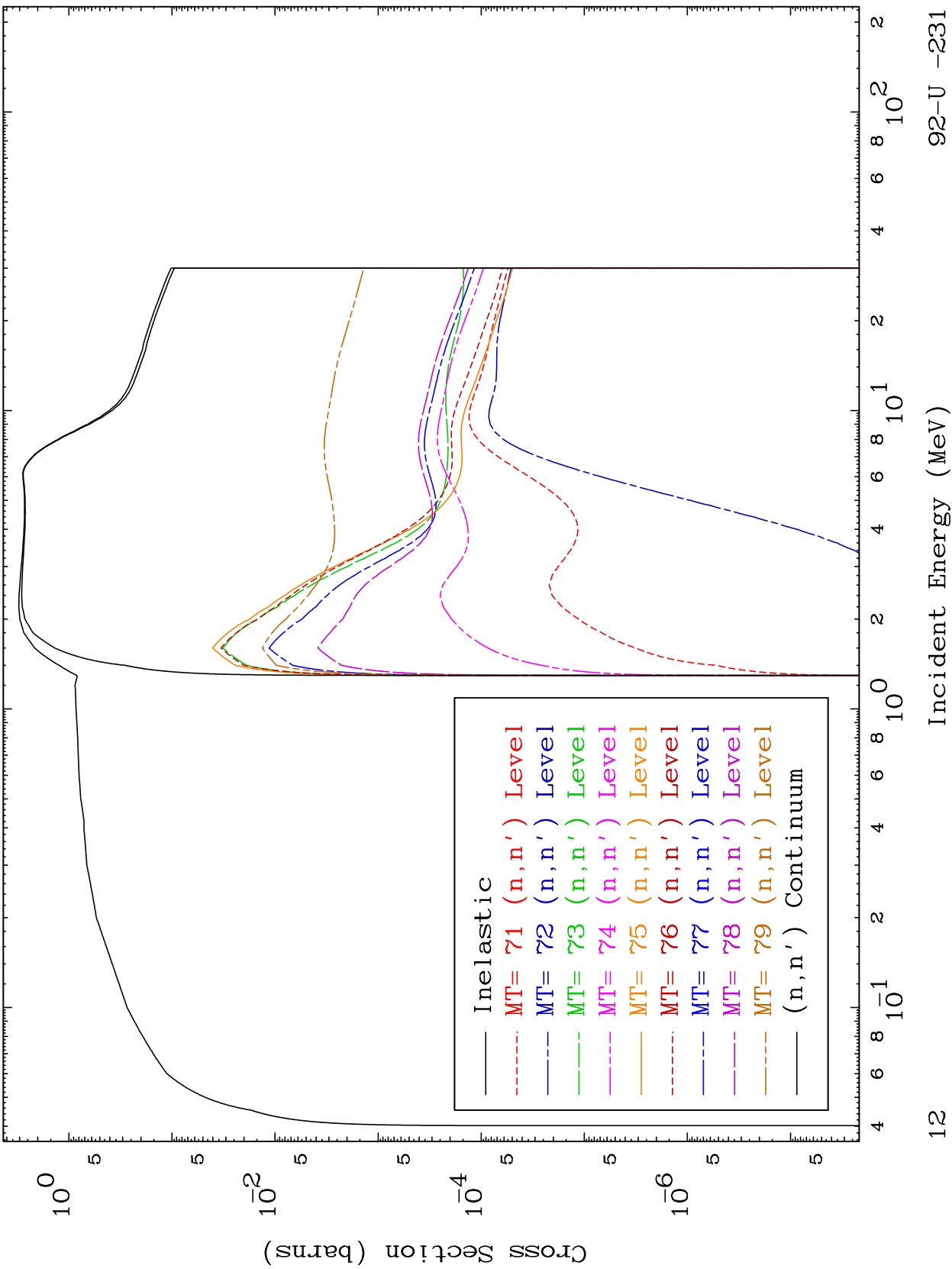
(n,n') Levels
293 Kelvin Cross Sections



MAT 9216

92-U -231

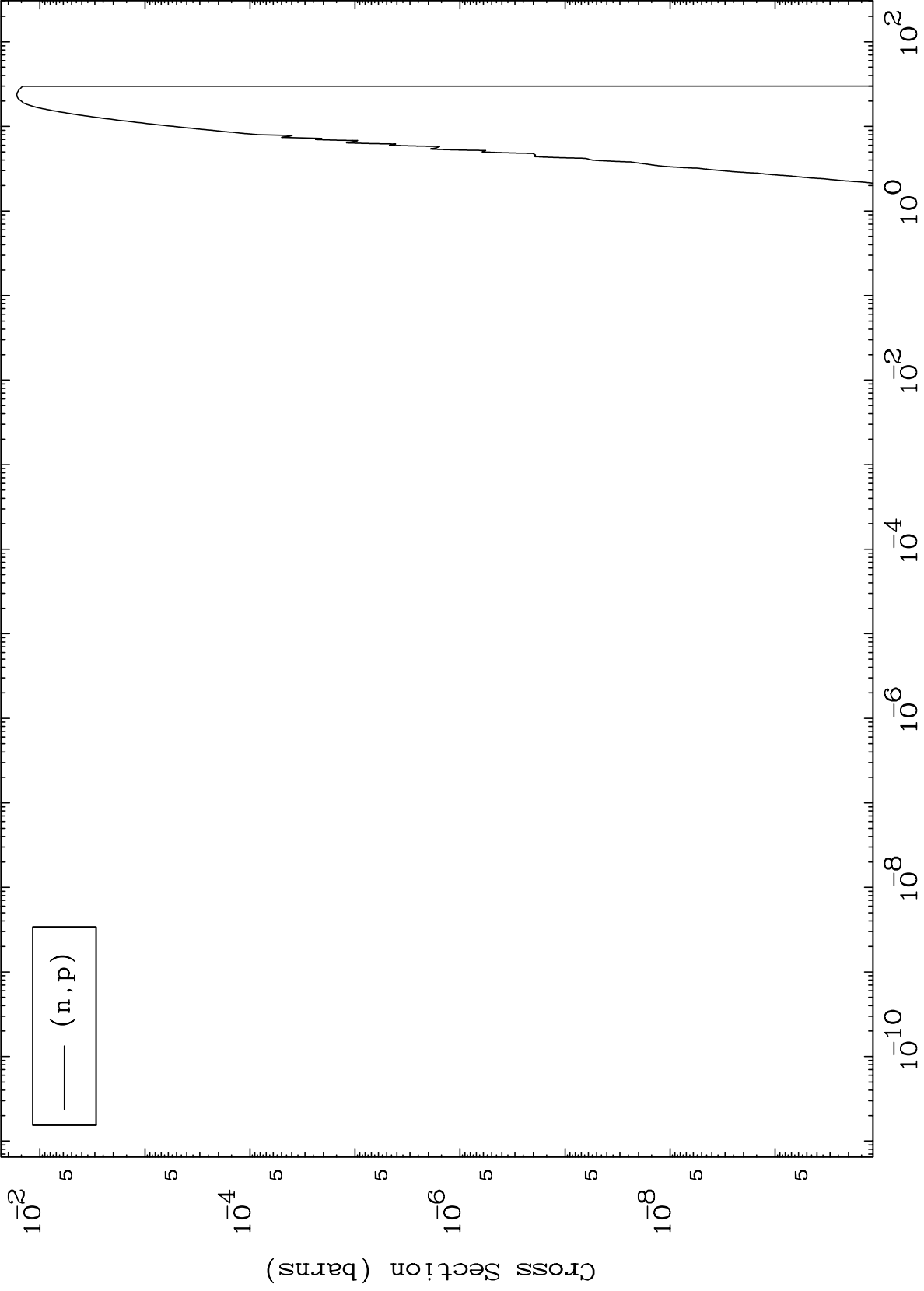
(n,n') Levels
293 Kelvin Cross Sections



MAT 9216

(n,p) Levels
293 Kelvin Cross Sections

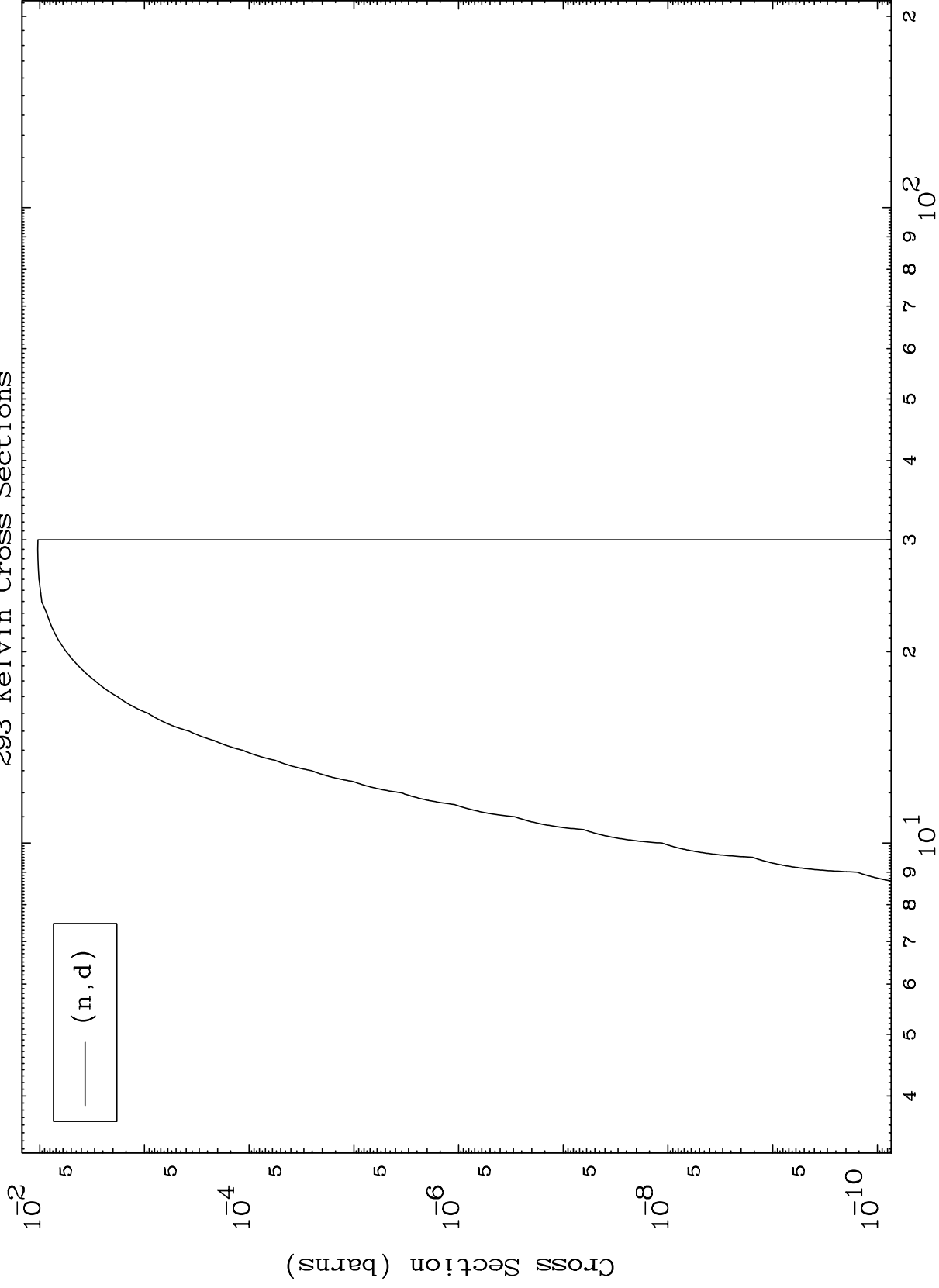
92-U -231



MAT 9216

(n,d) Levels
293 Kelvin Cross Sections

92-U -231



14

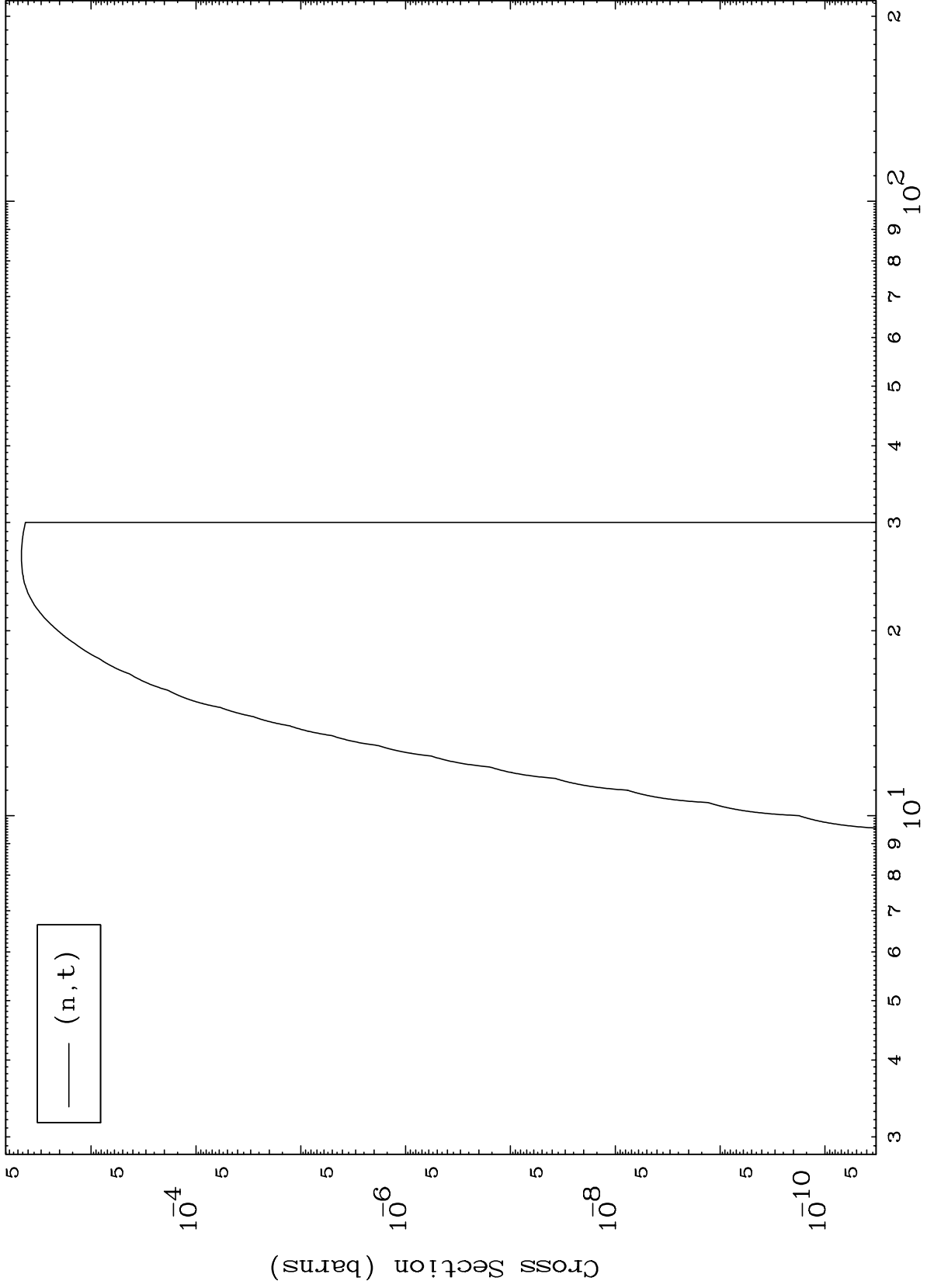
Incident Energy (MeV)

92-U -231

MAT 9216

(n,t) Levels
293 Kelvin Cross Sections

92-U -231



(n,t)

15

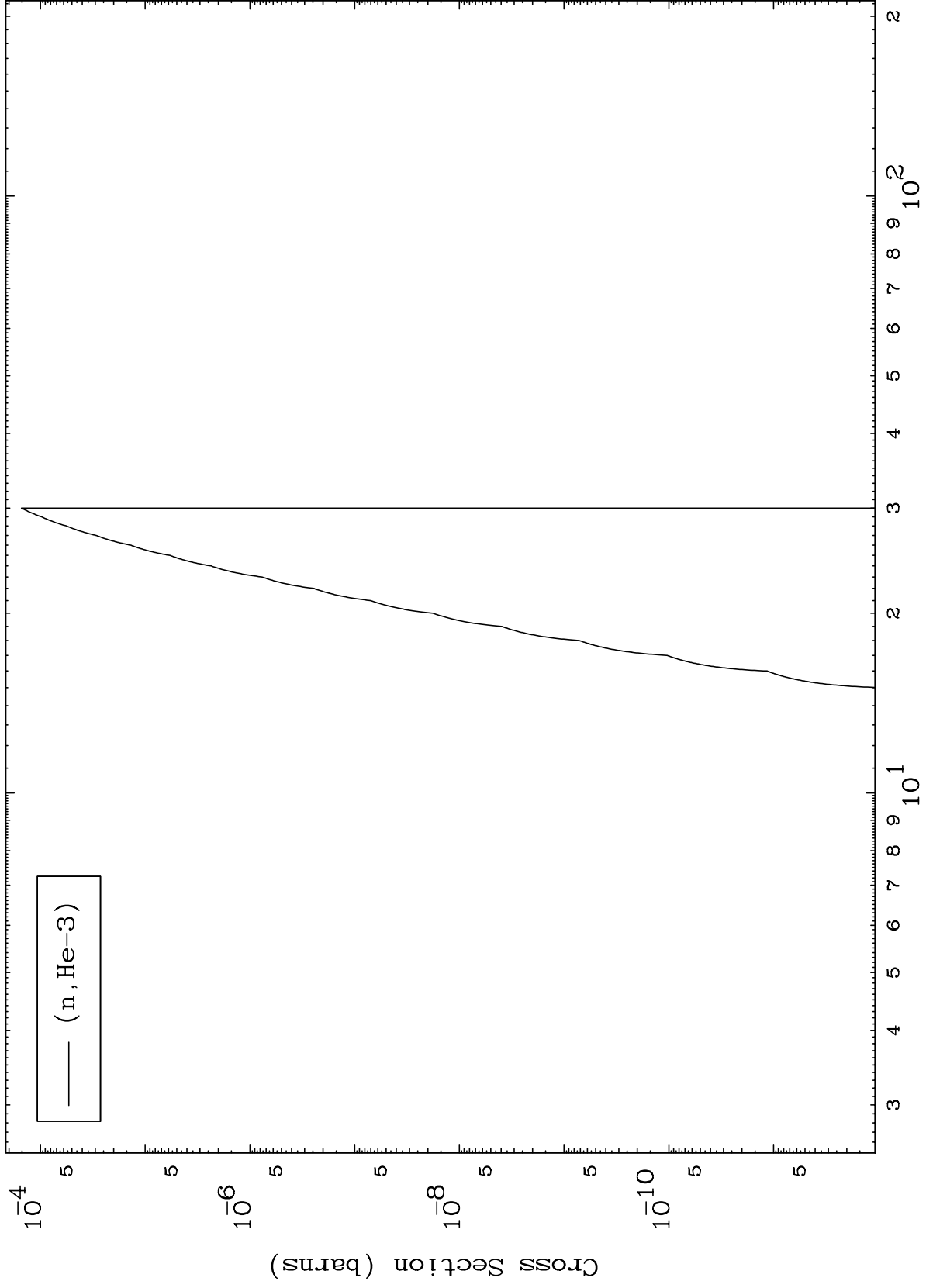
Incident Energy (MeV)

92-U -231

MAT 9216

(n,He3) Levels
293 Kelvin Cross Sections

92-U -231



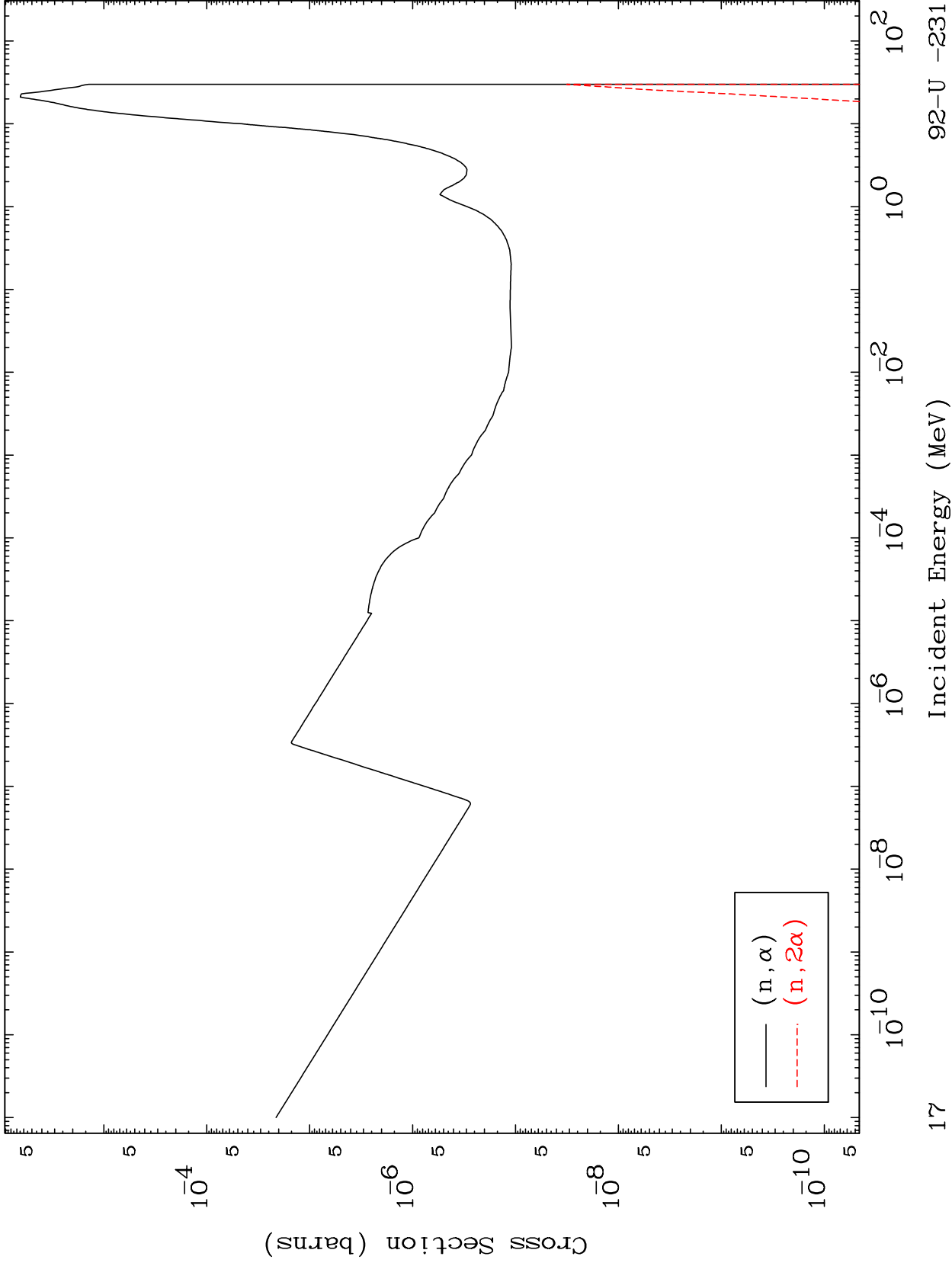
16

92-U -231

MAT 9216

(n,α) Levels
293 Kelvin Cross Sections

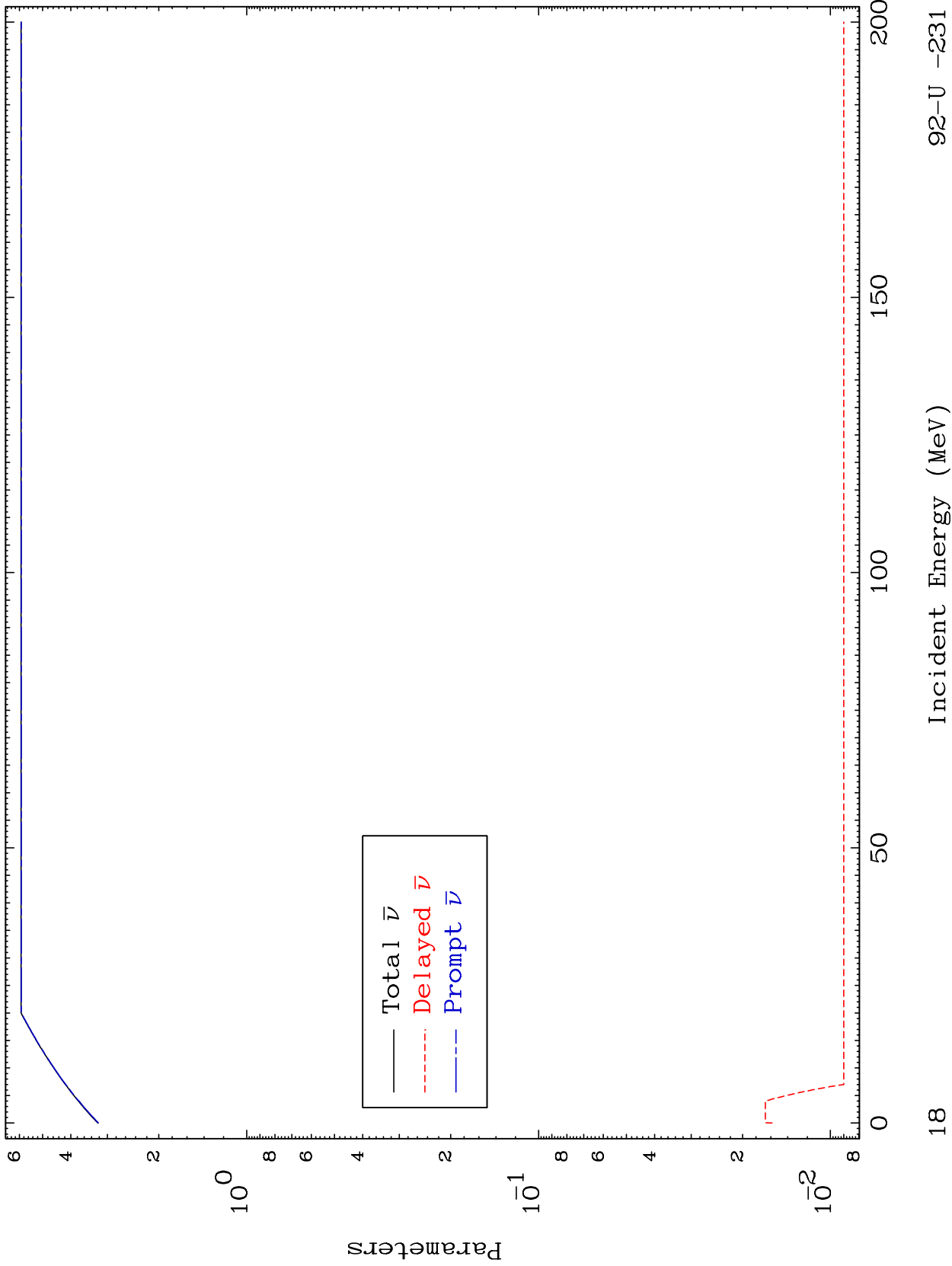
92-U -231



MAT 9216

Energy Release
Parameters

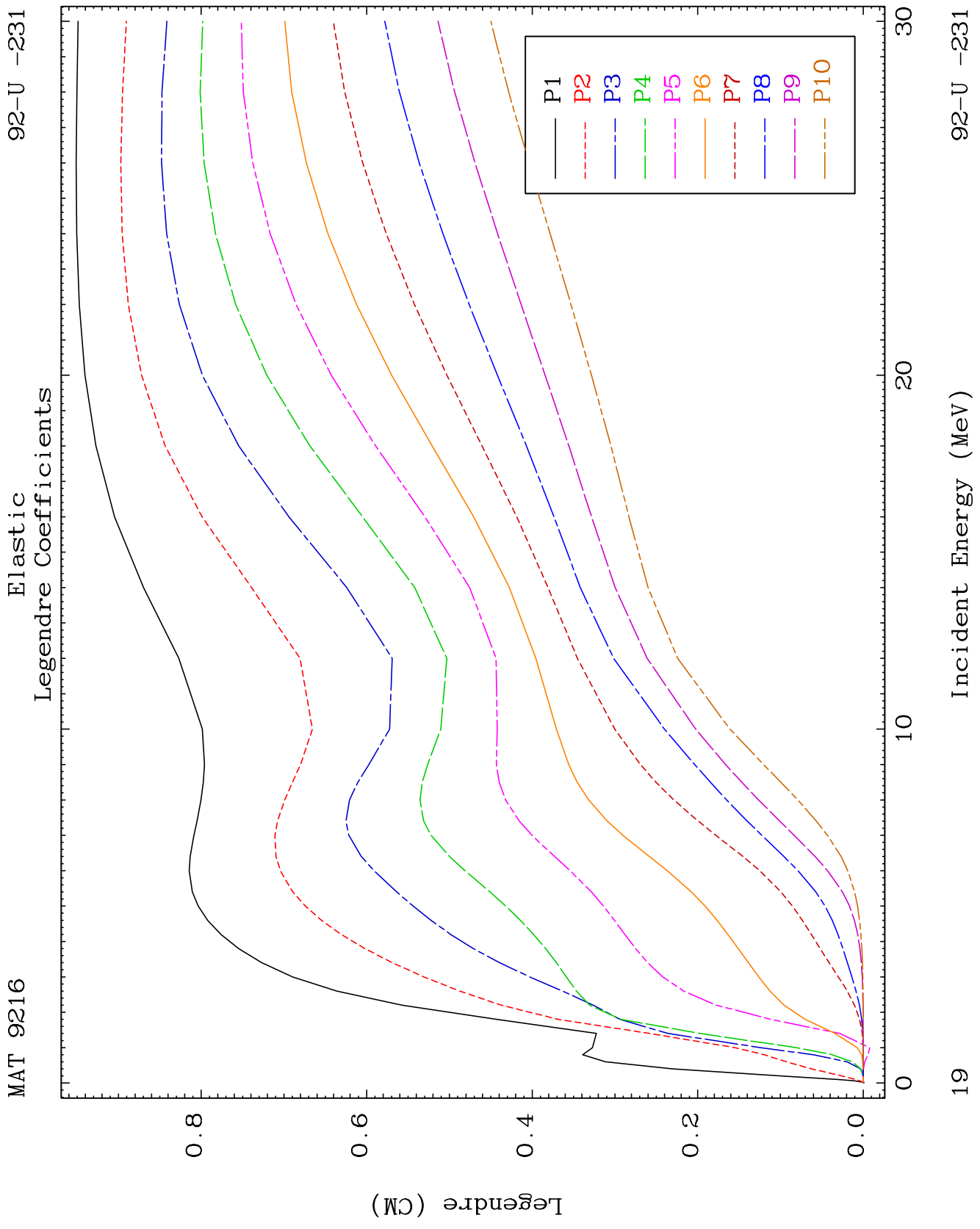
92-U -231



18

Incident Energy (MeV)

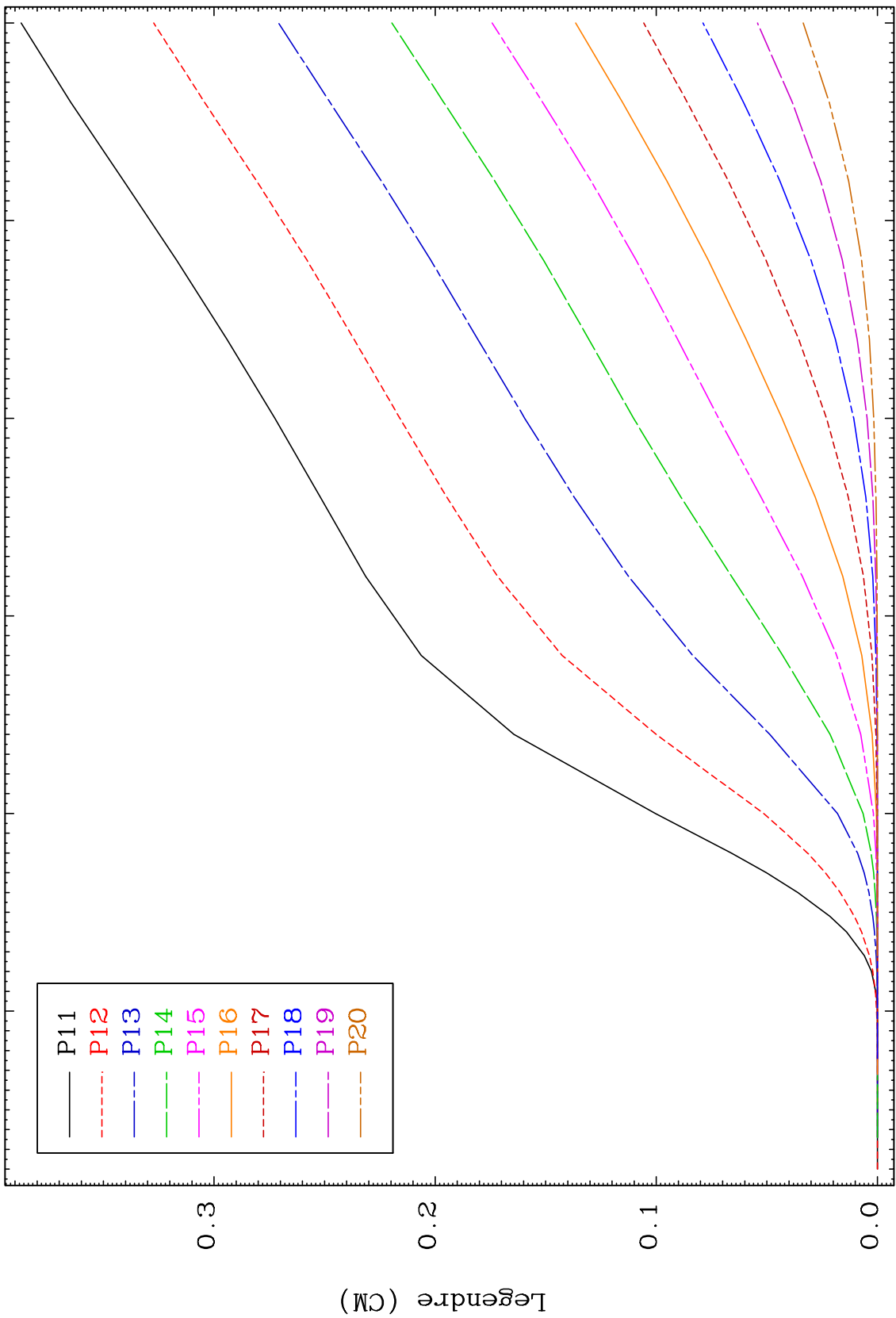
92-U -231



MAT 9216

Elastic Legendre Coefficients

92-U -231



20

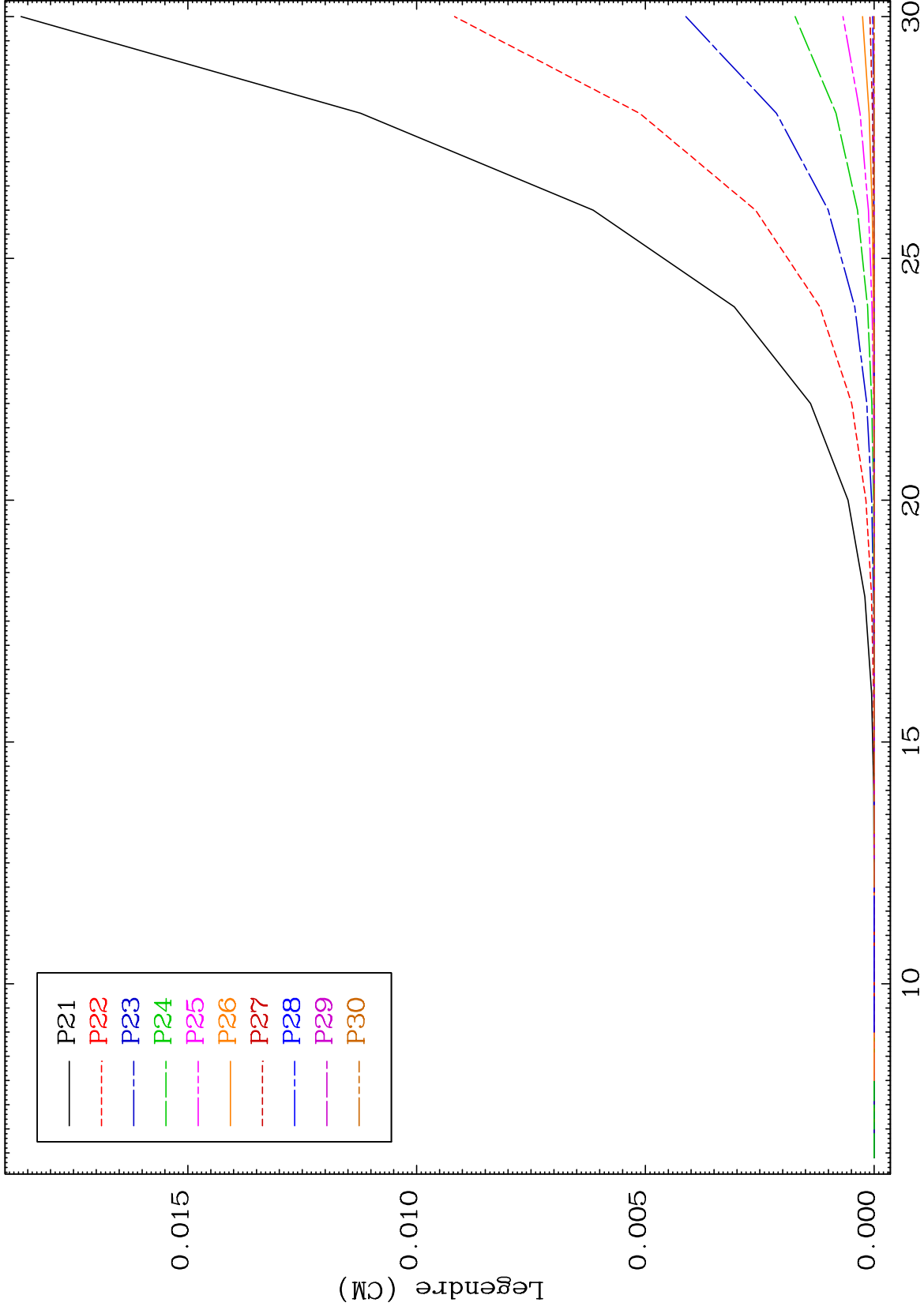
Incident Energy (MeV)

92-U -231

MAT 9216

Elastic
Legendre Coefficients

92-U -231



21

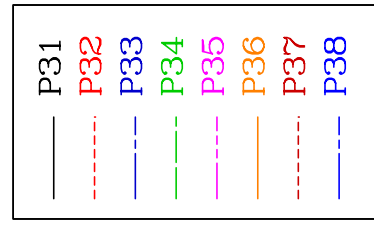
Incident Energy (MeV)

92-U -231

MAT 9216

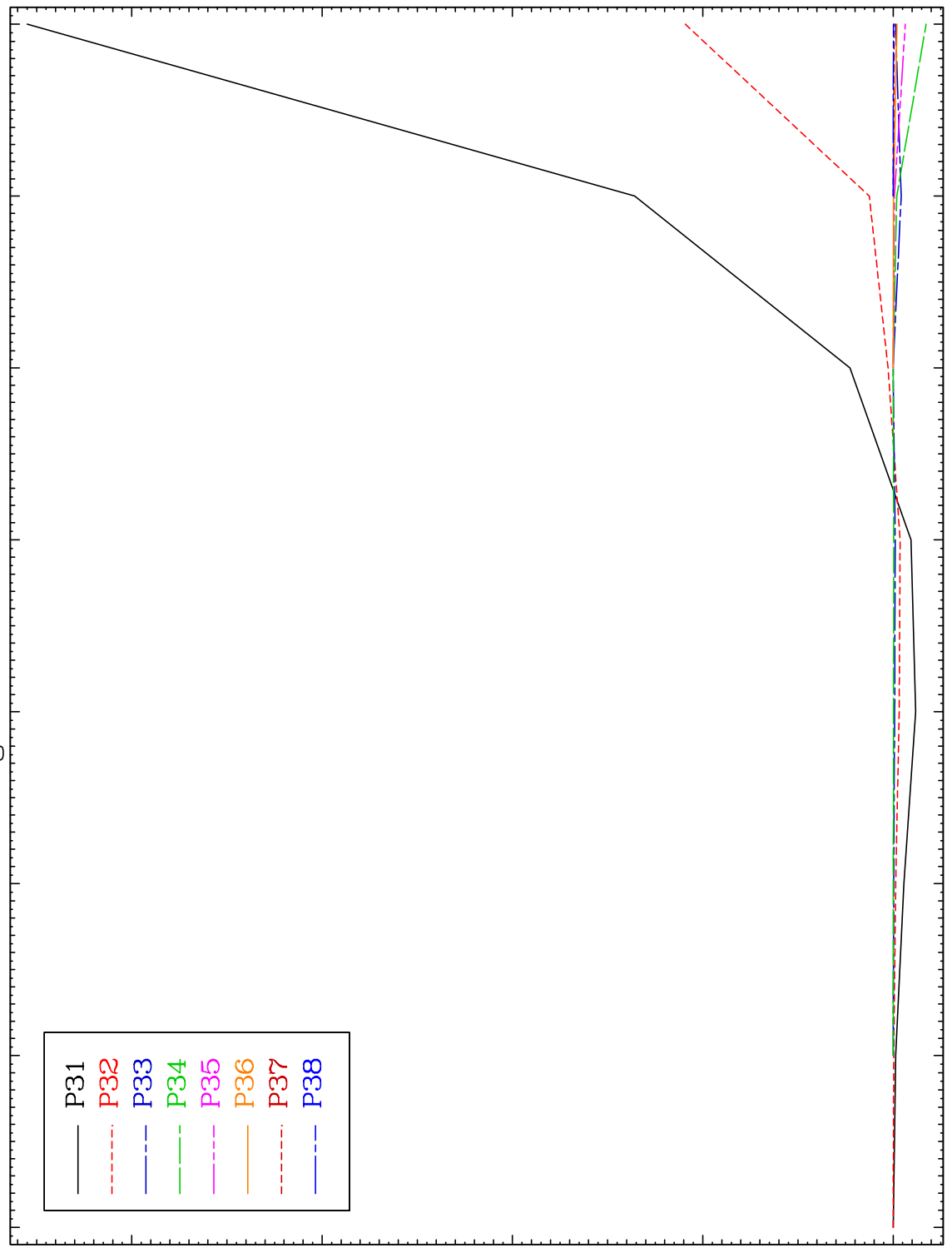
Elastic
Legendre Coefficients

92-U -231



$\times 10^{-7}$

Legendre (CM)



16

18

20

22

24

26

28

30

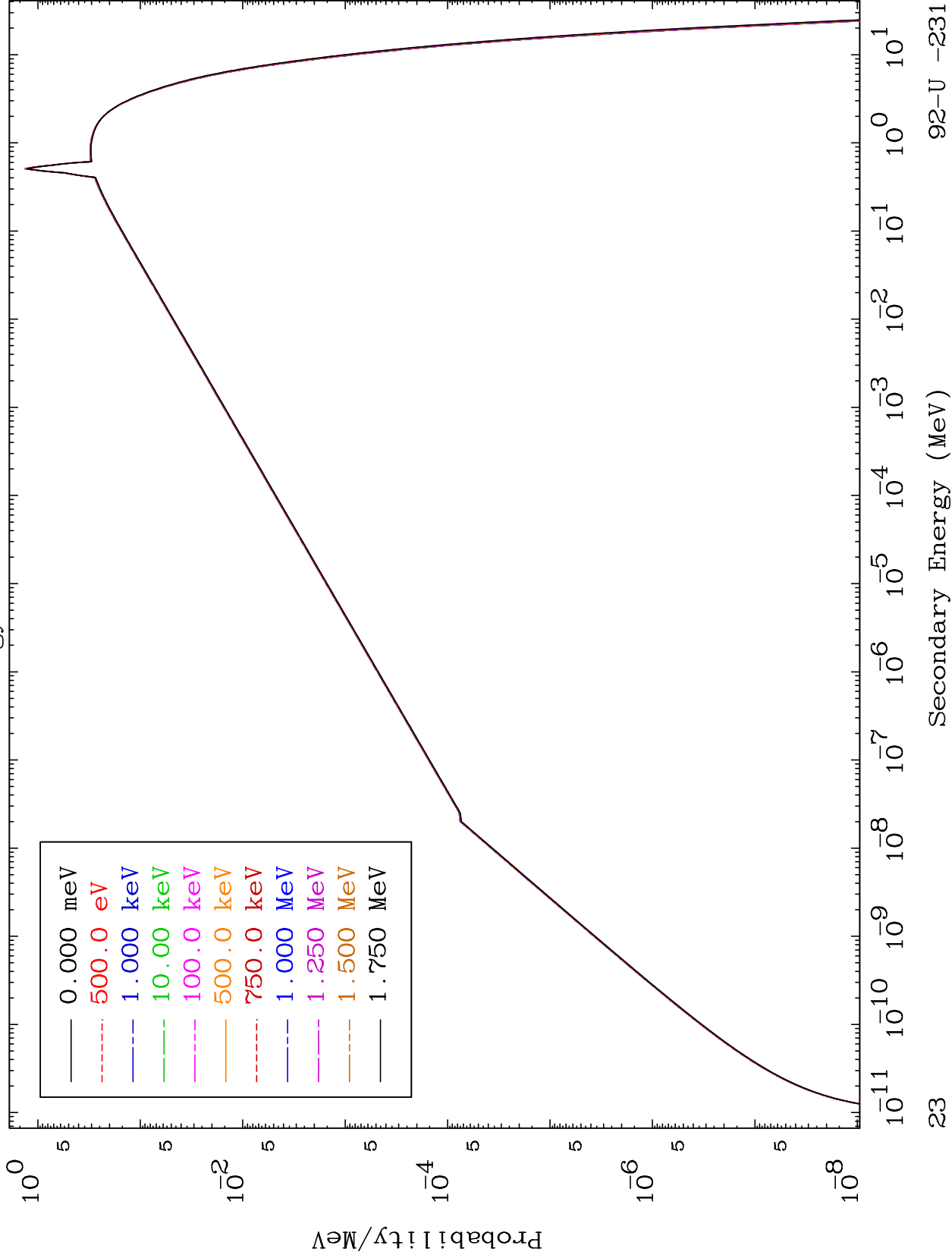
92-U -231

Incident Energy (MeV)

MAT 9216

Fission Energy Distributions

92-U -231



92-U -231

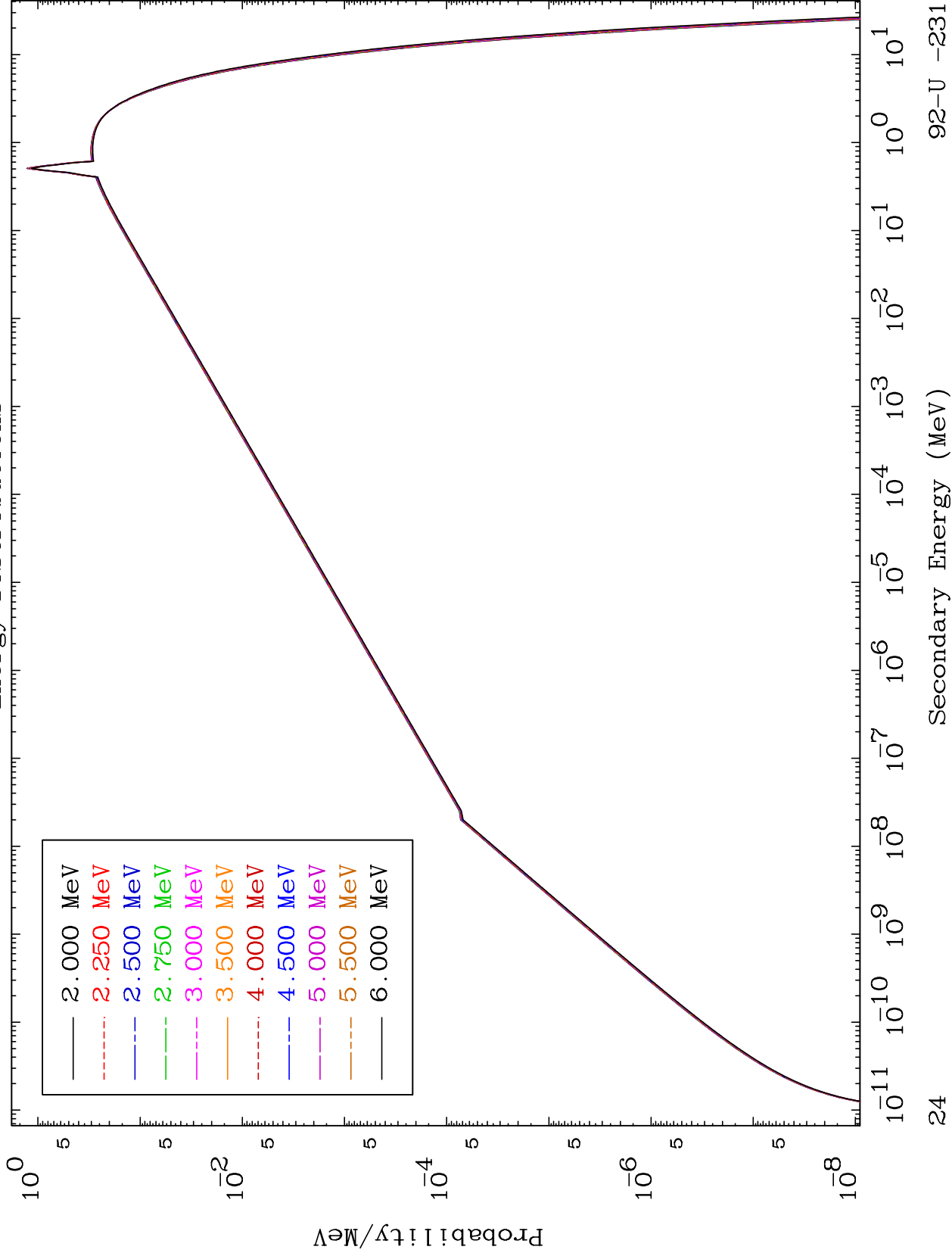
Secondary Energy (MeV)

23

MAT 9216

Fission Energy Distributions

92-U -231



24

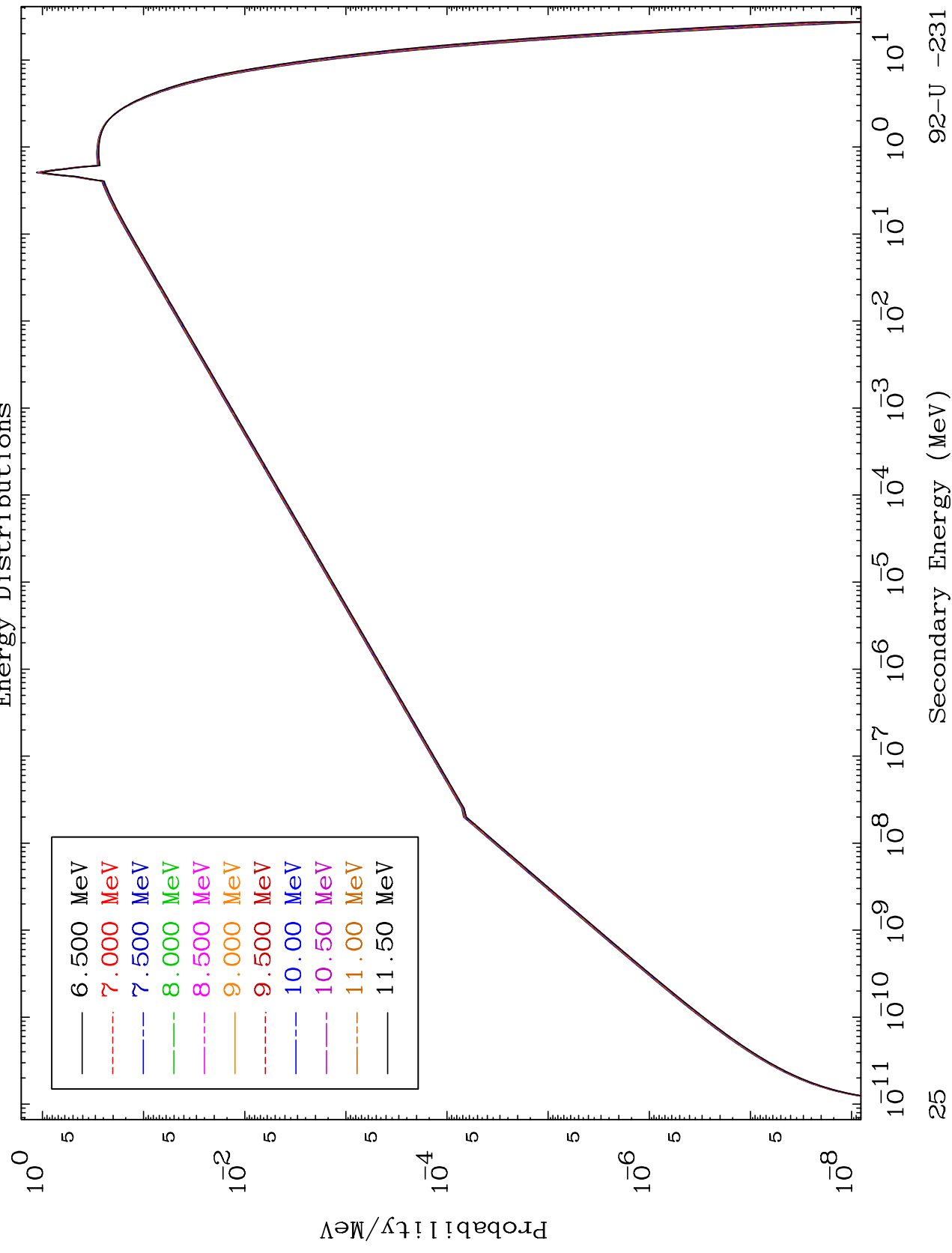
Secondary Energy (MeV)

92-U -231

MAT 9216

Fission Energy Distributions

92-U -231



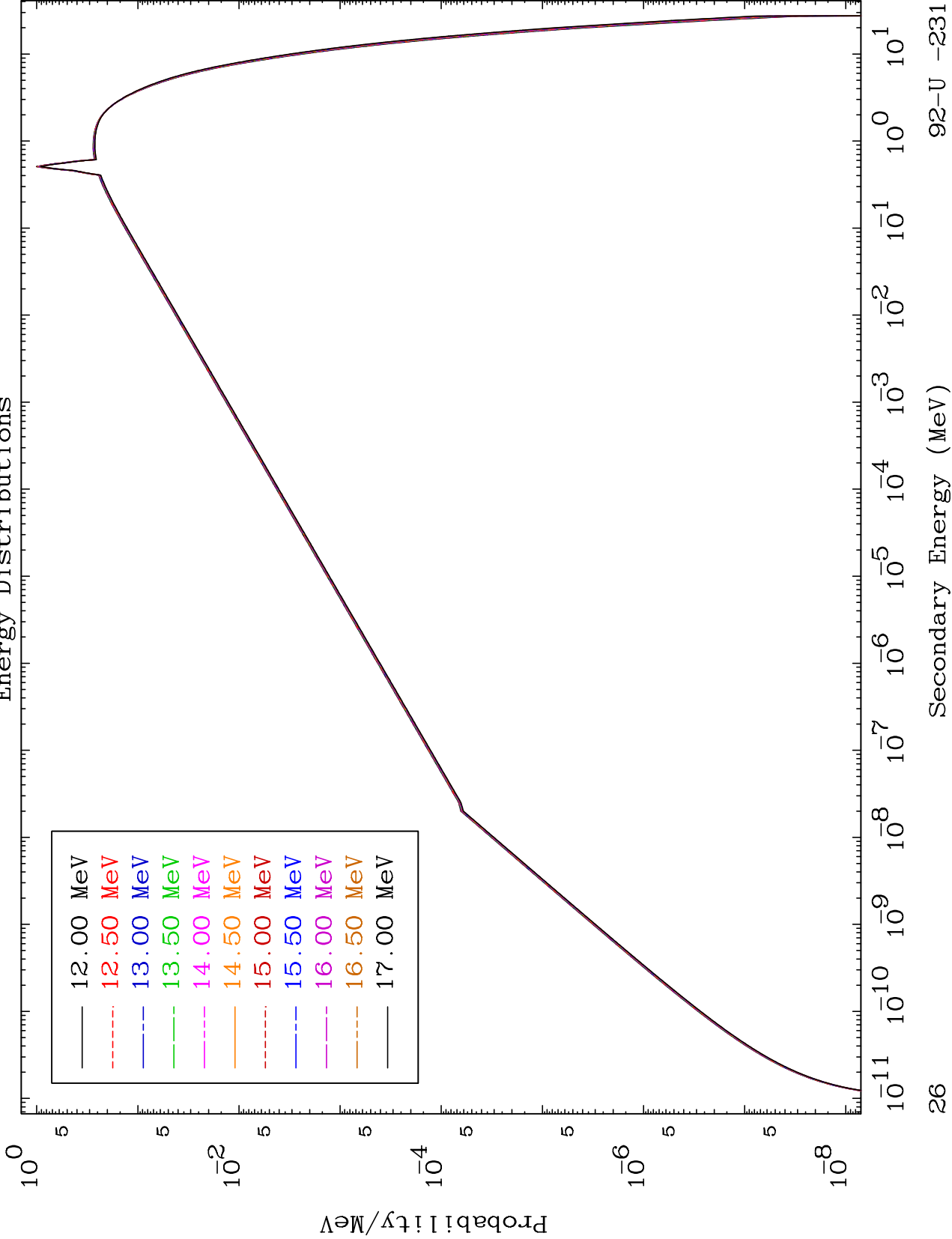
25

92-U -231

MAT 9216

Fission Energy Distributions

92-U -231



26

Secondary Energy (MeV)

92-U -231

