

Program EVALPLOT  
(Version 2021-1)

by

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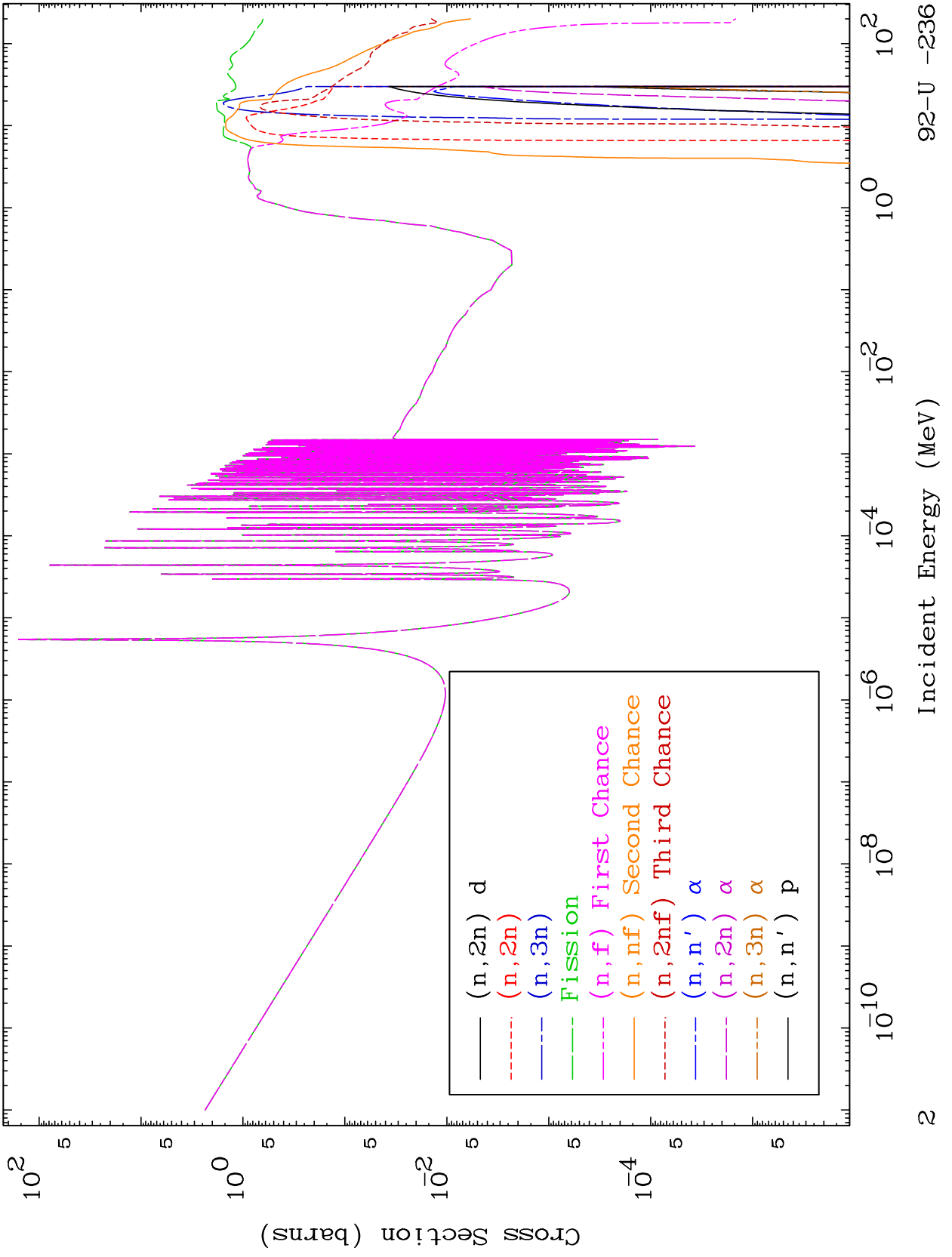
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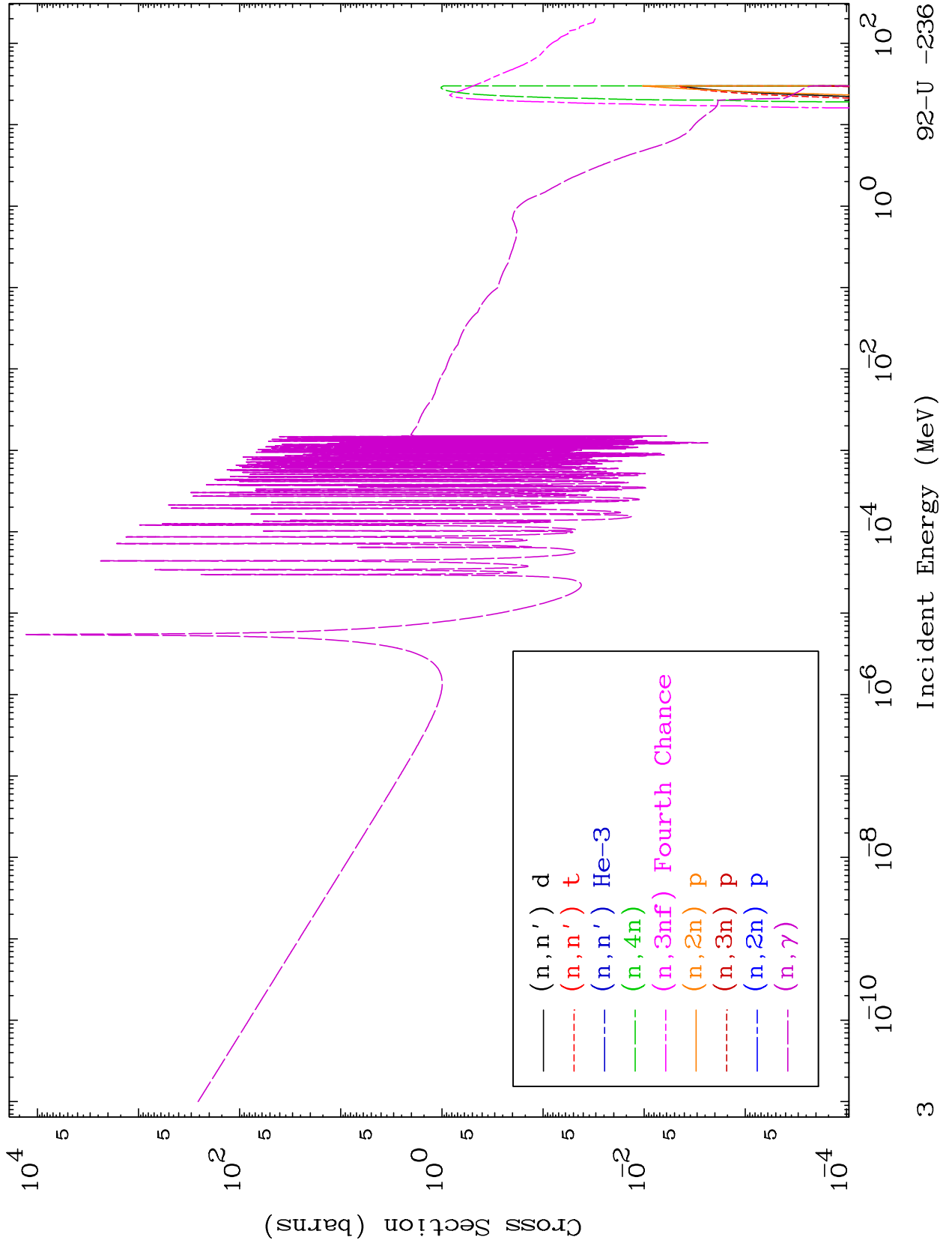


MAT 9231

Neutron Absorption  
293 Kelvin Cross Sections

92-U -236

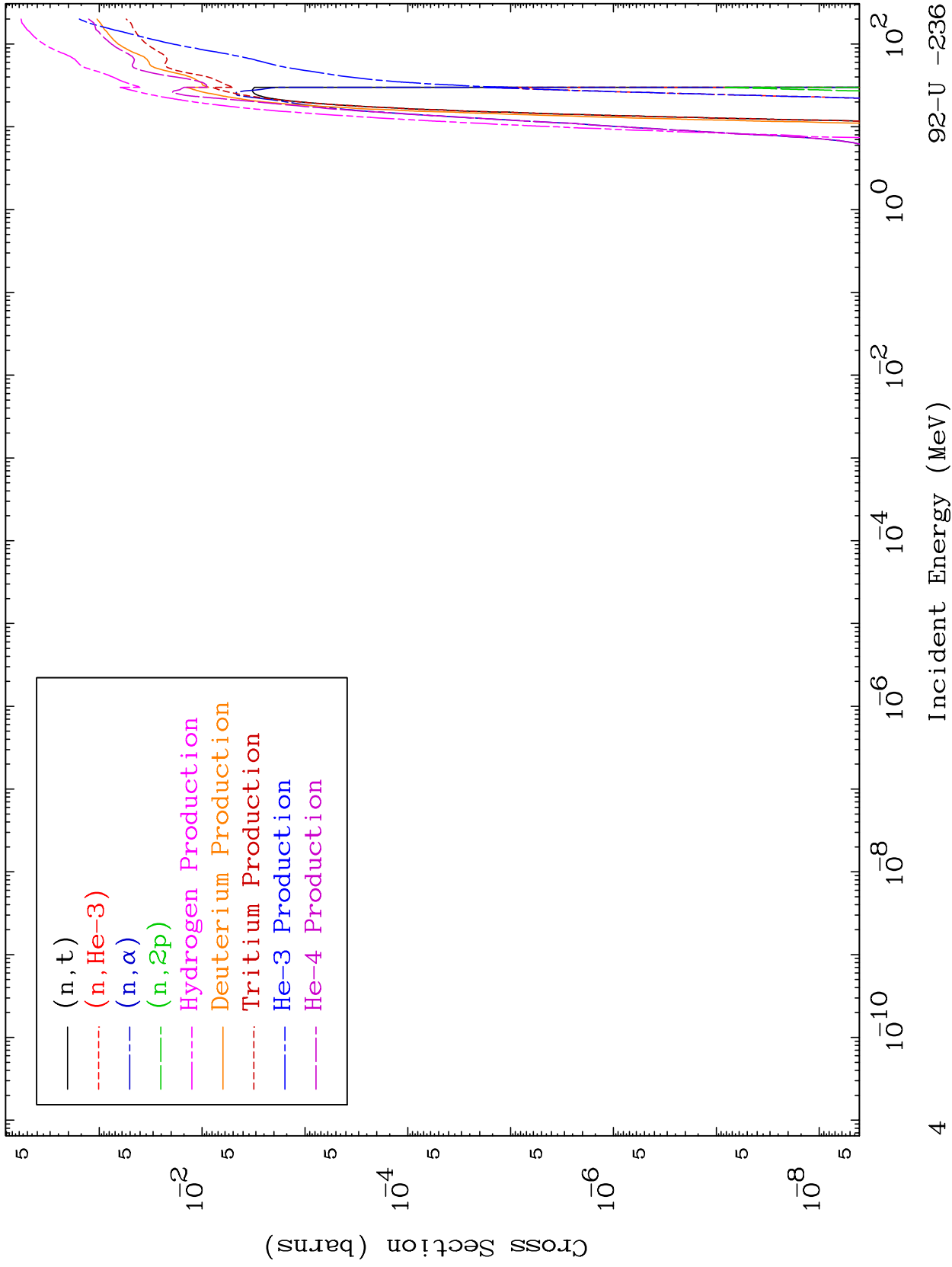




MAT 9231

Neutron Absorption  
293 Kelvin Cross Sections

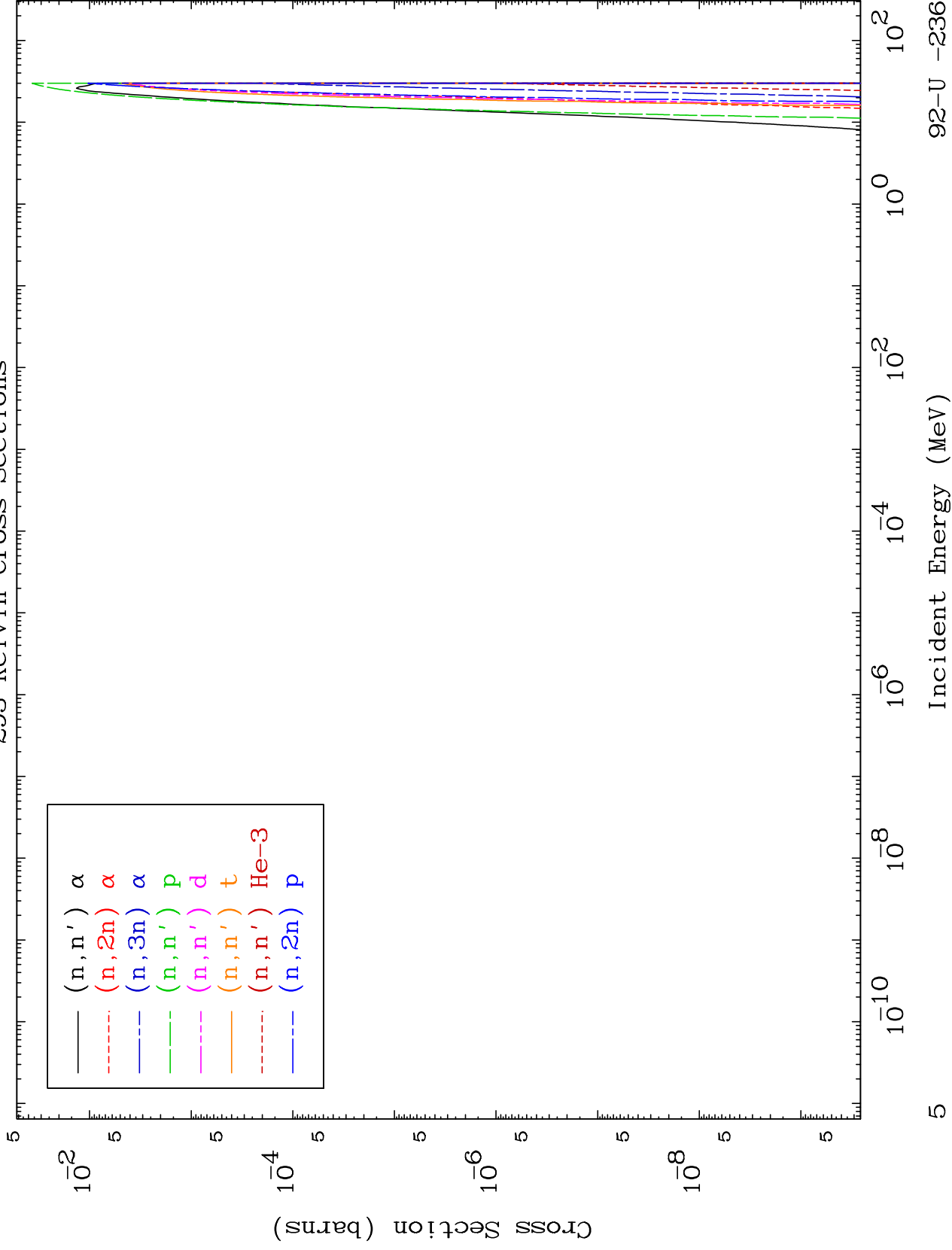
92-U -236



MAT 9231

Charged Particle  
293 Kelvin Cross Sections

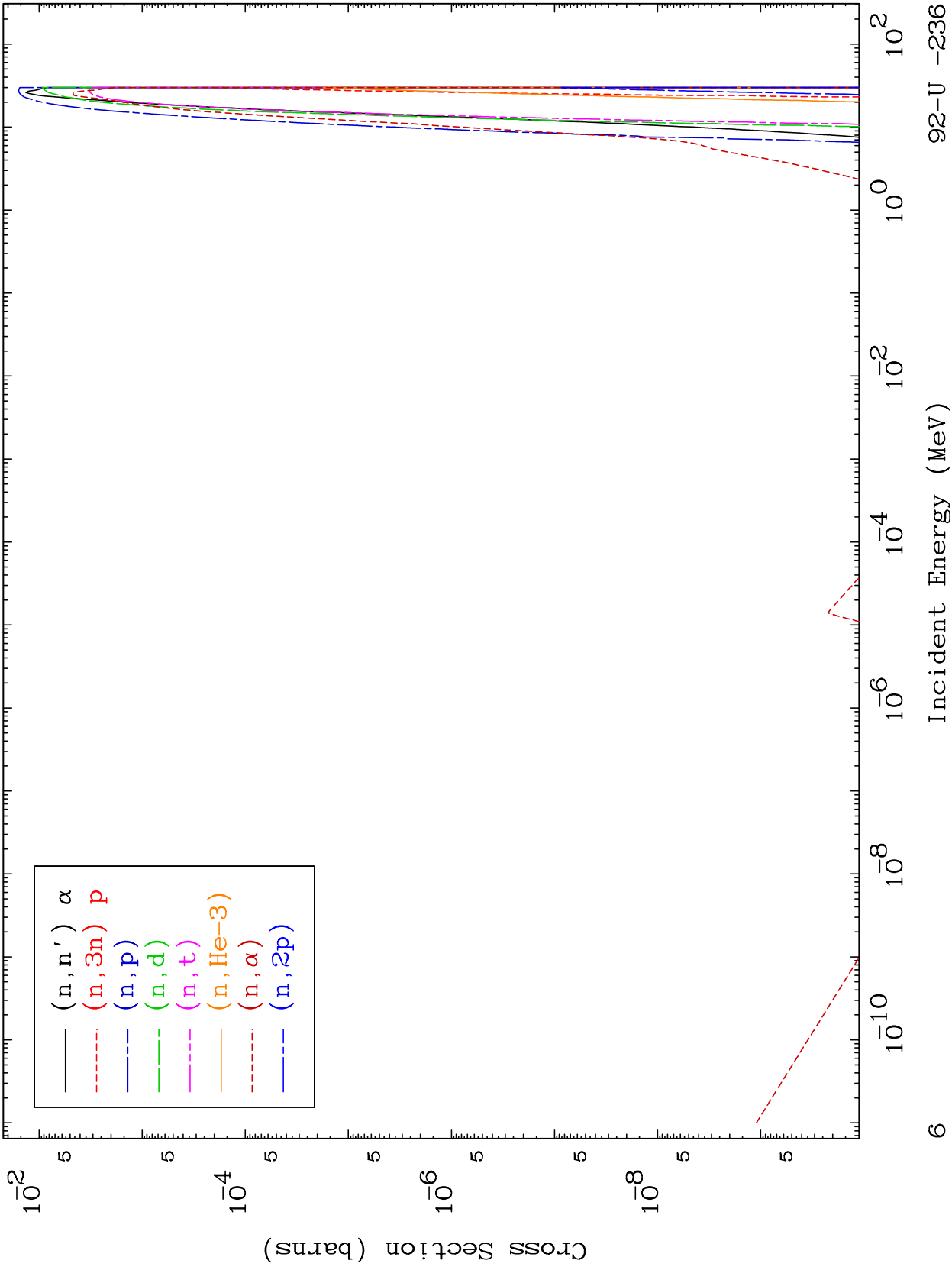
92-U -236

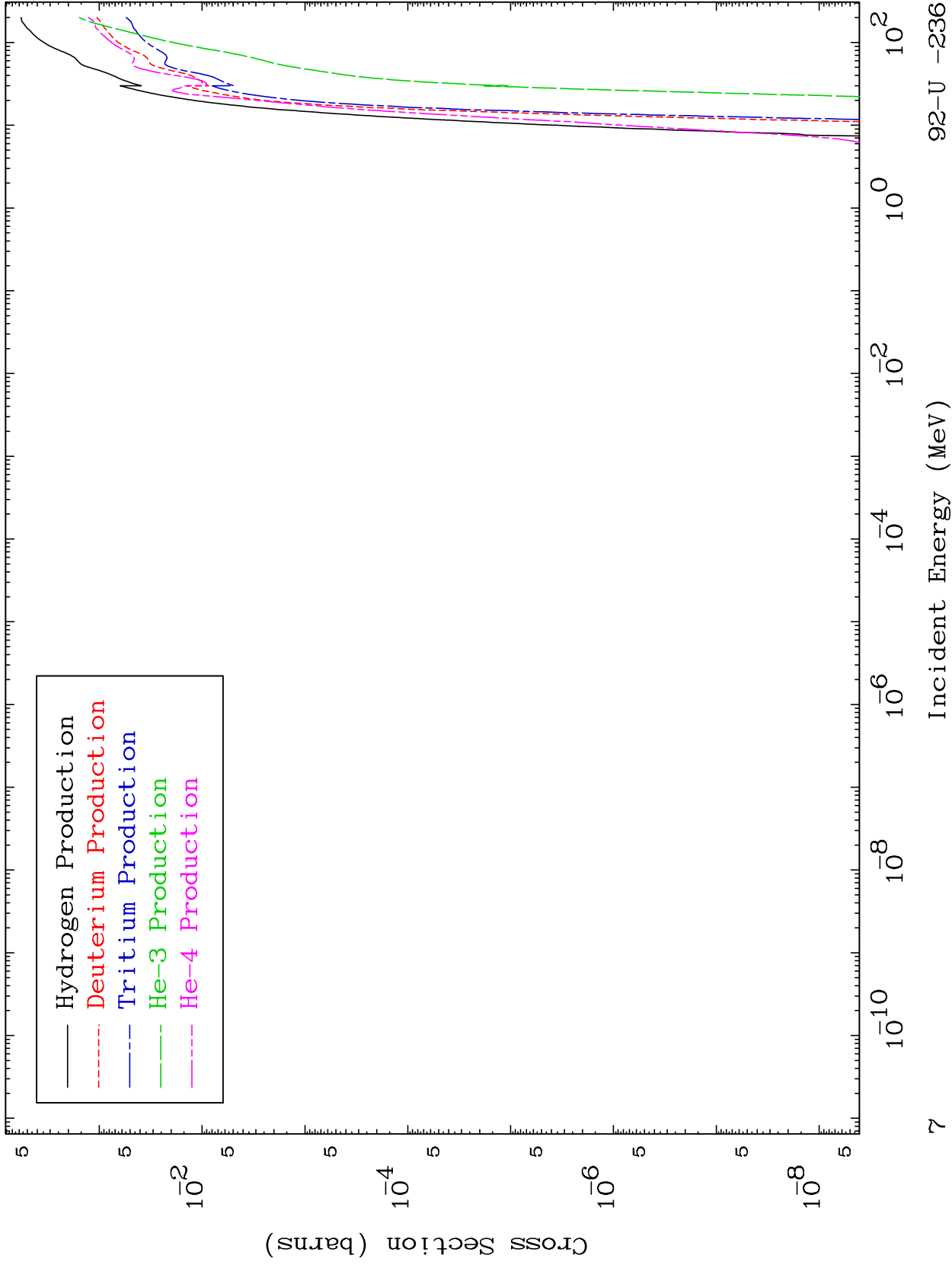


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Charged Particle  
293 Kelvin Cross Sections

92-U -236



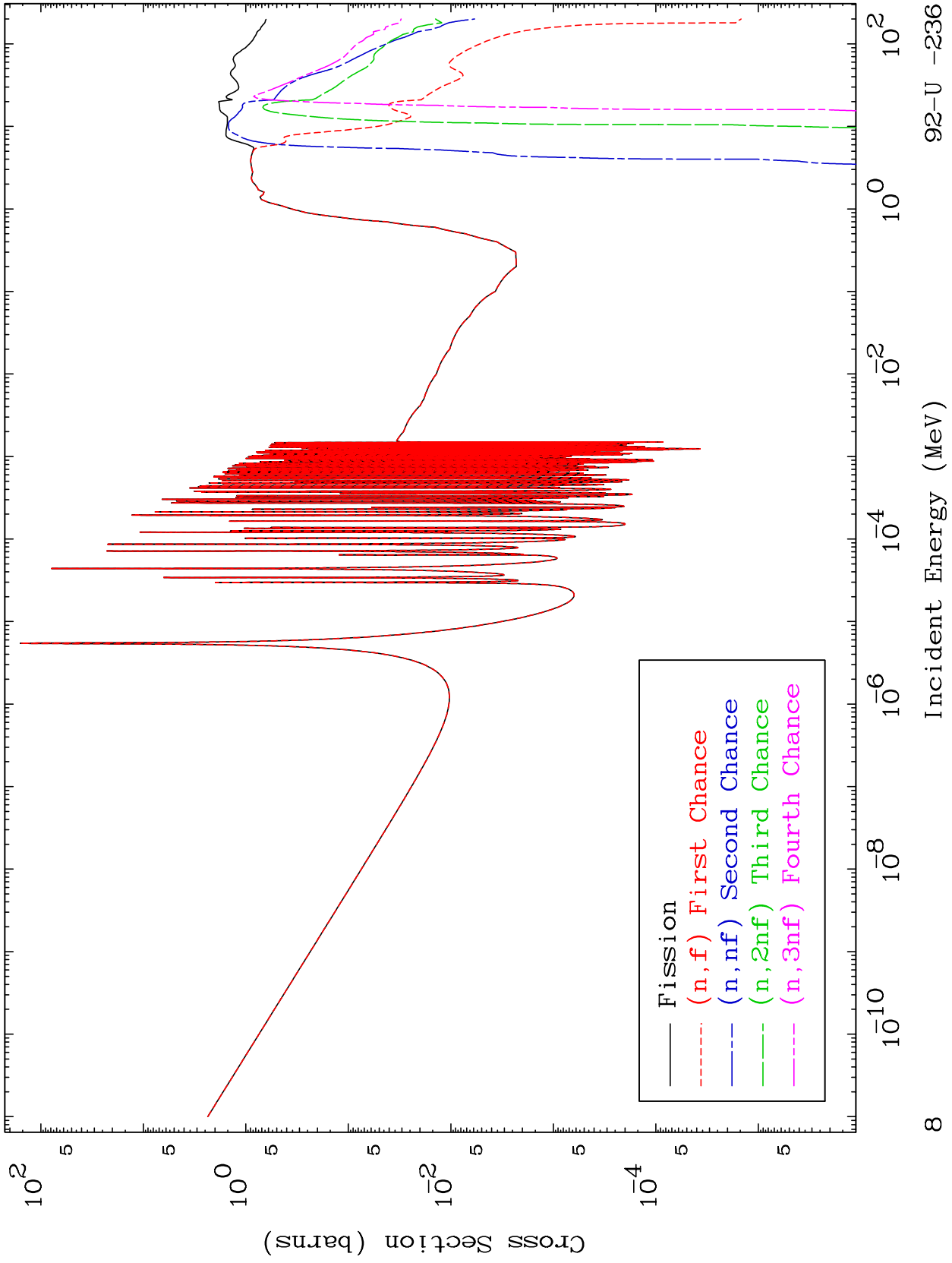




MAT 9231

Fission  
293 Kelvin Cross Sections

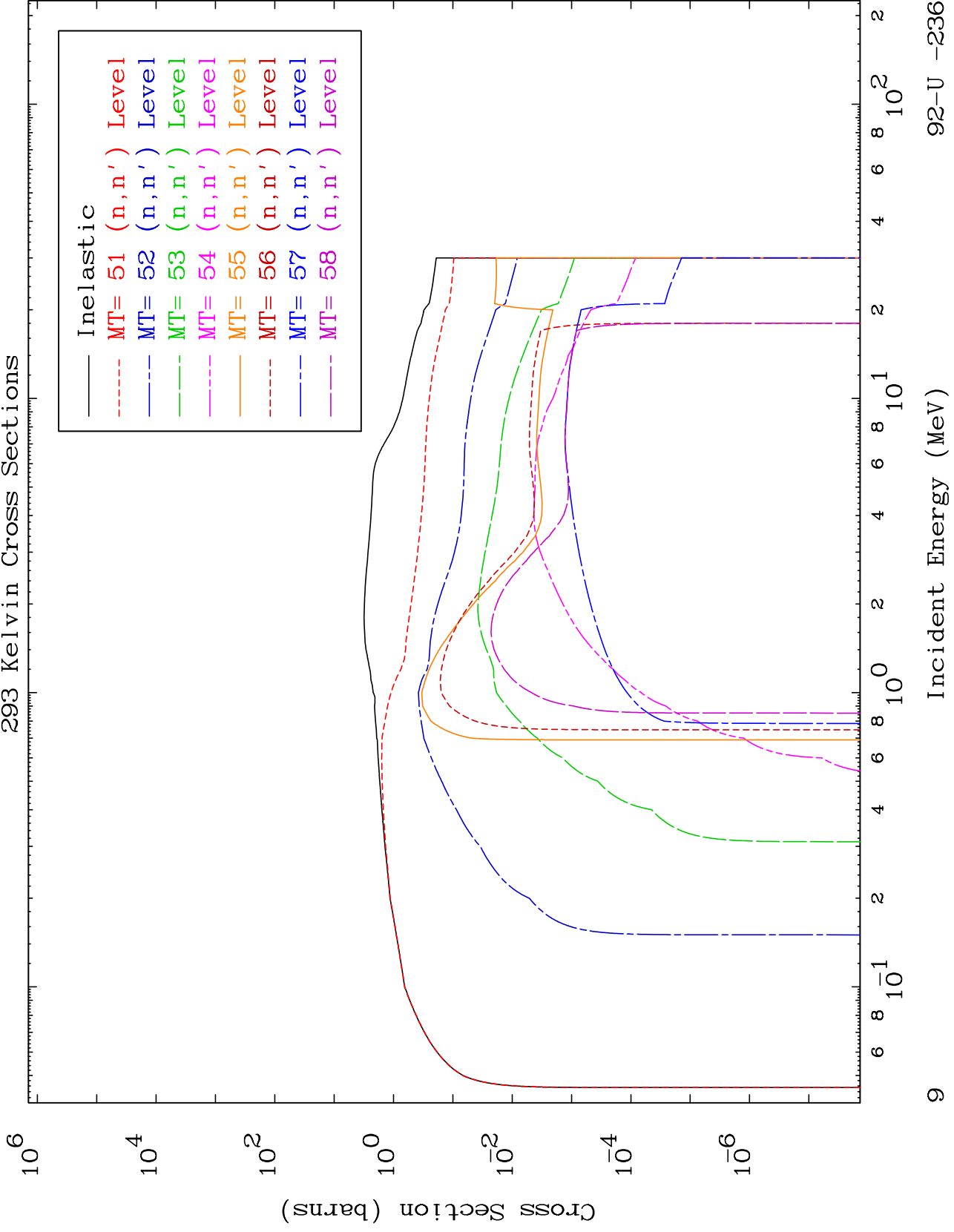
92-U -236



MAT 9231

(n,n') Levels  
293 Kelvin Cross Sections

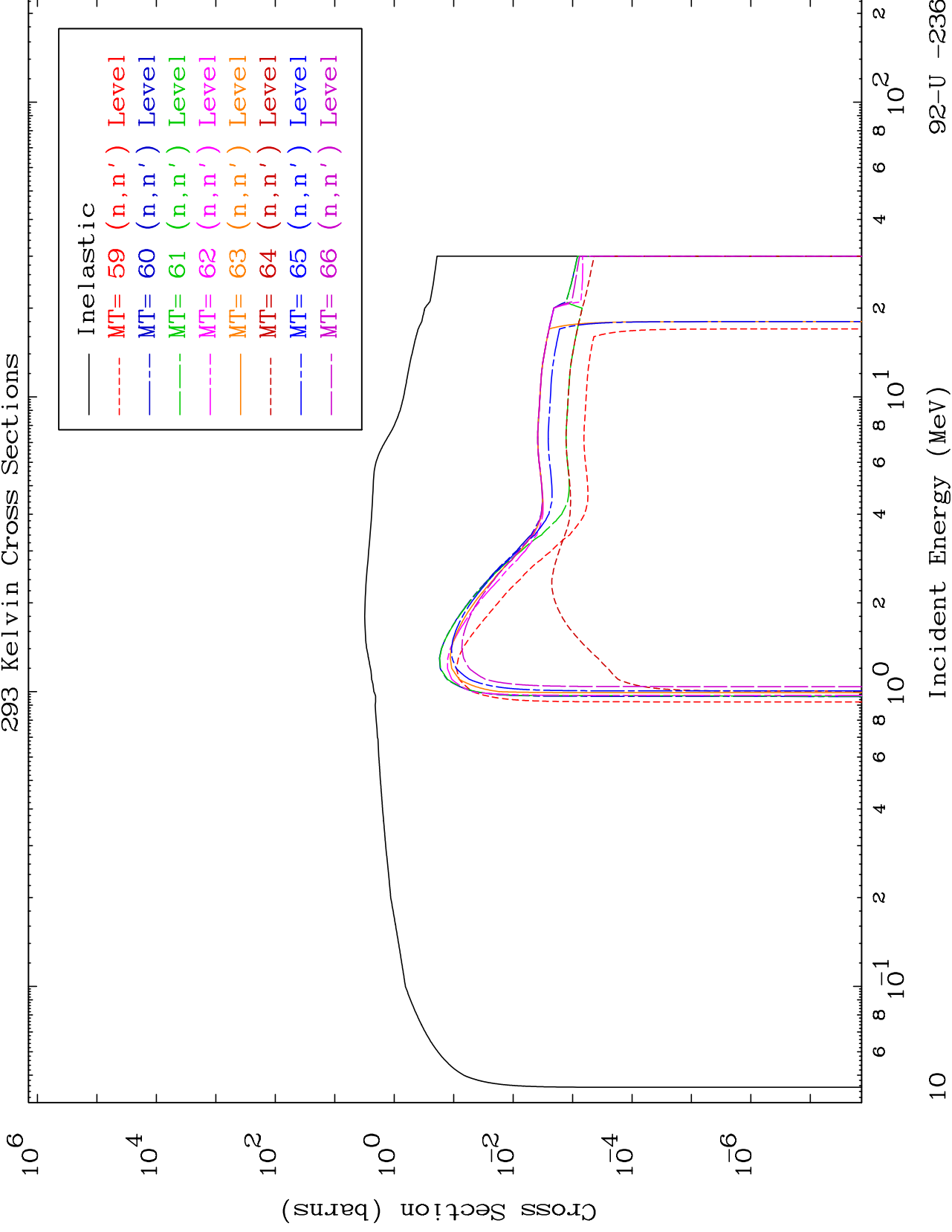
92-U -236



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(n,n') Levels  
293 Kelvin Cross Sections

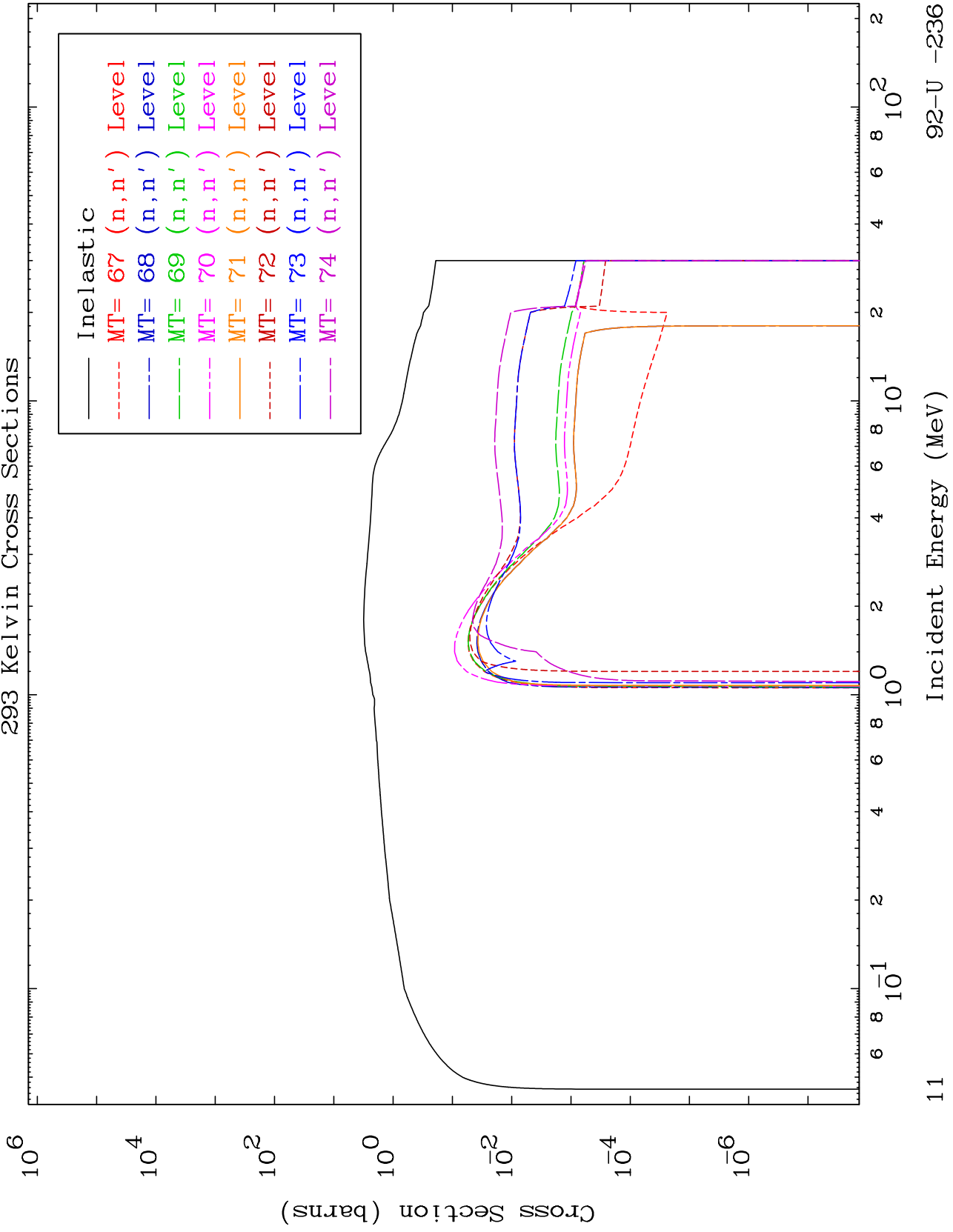
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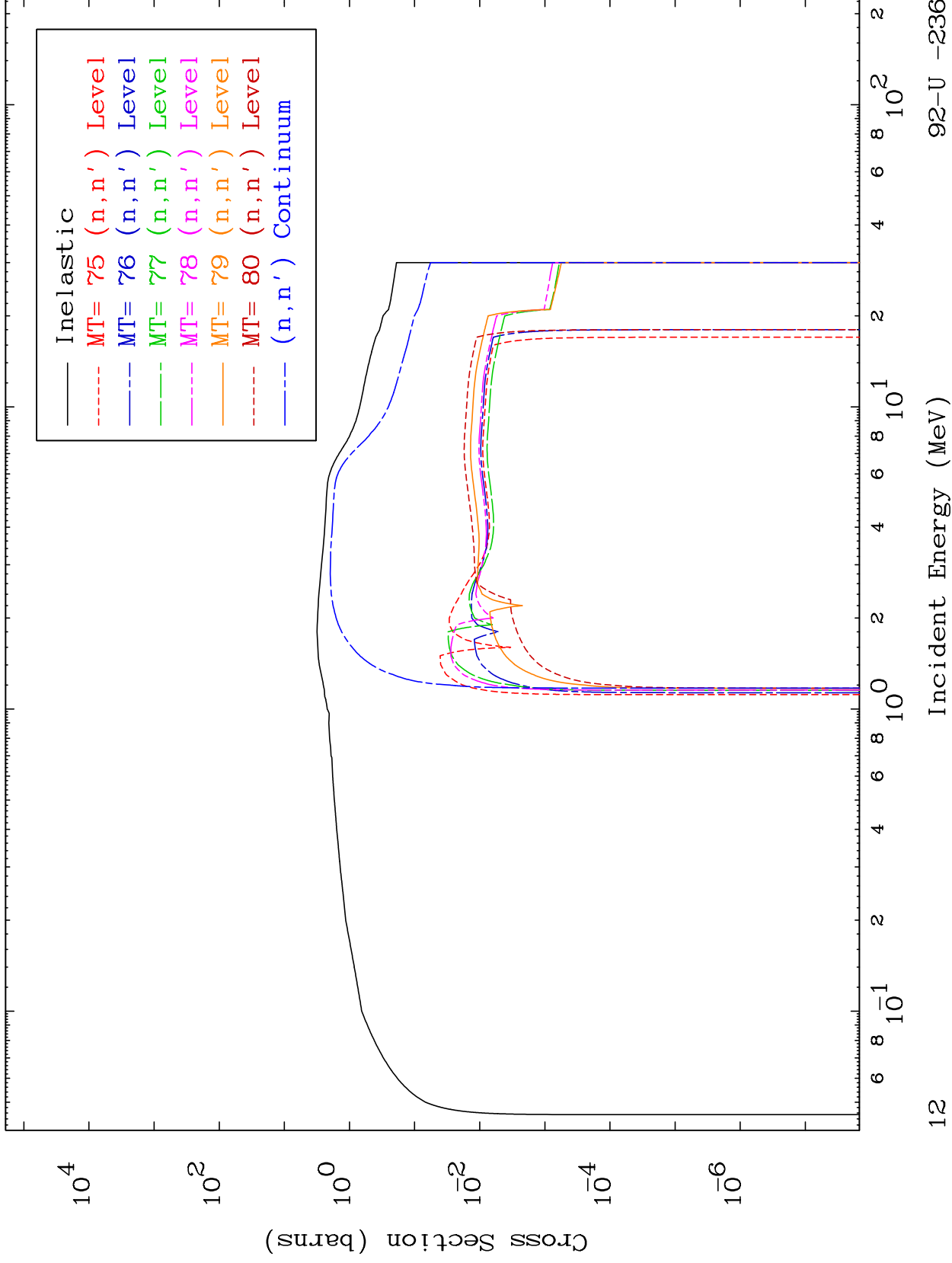


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Incident Energy (MeV)

92-U -236

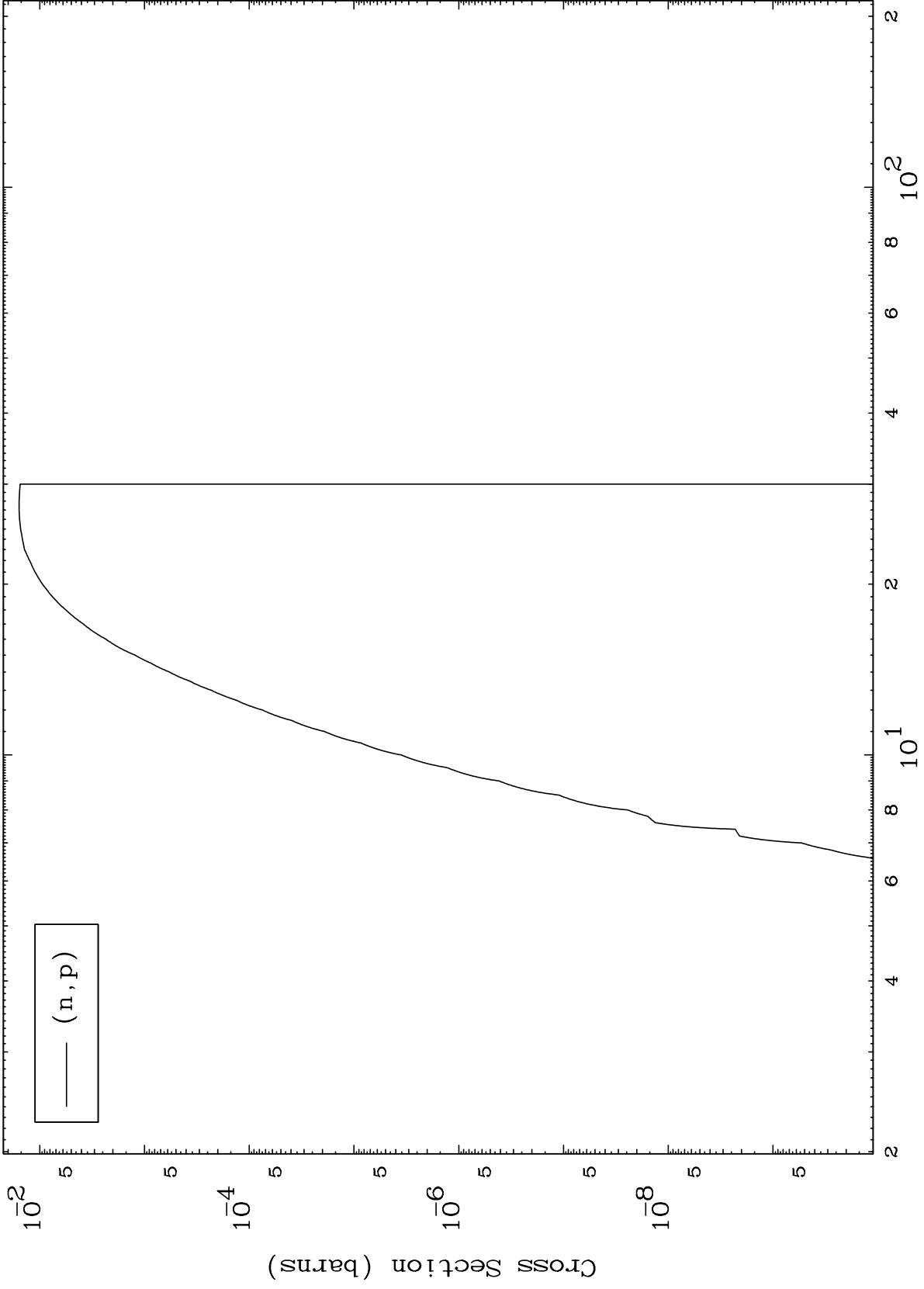




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(n,p) Levels  
293 Kelvin Cross Sections

92-U -236



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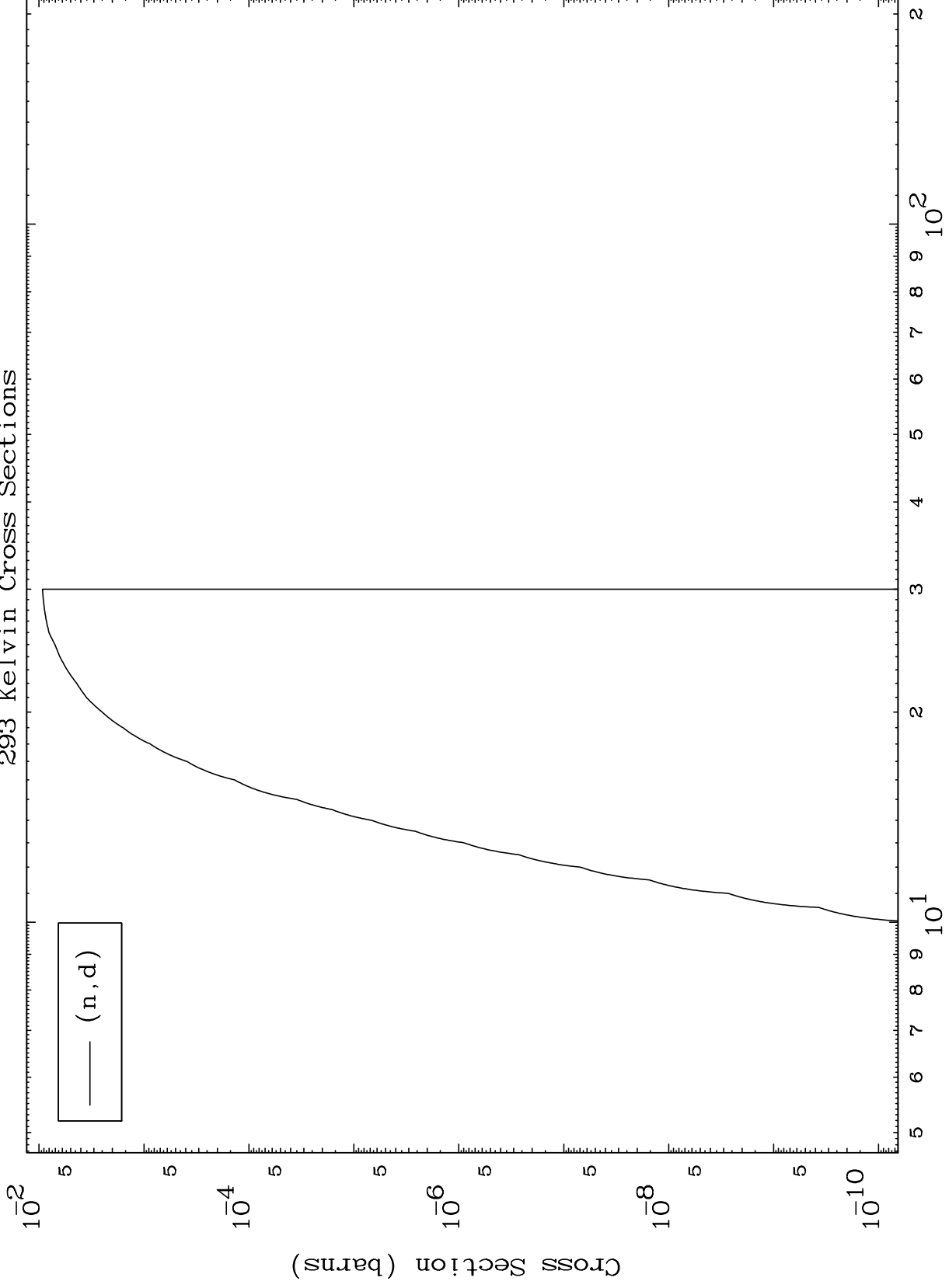
Incident Energy (MeV)

92-U -236

MAT 9231

(n,d) Levels  
293 Kelvin Cross Sections

92-U -236



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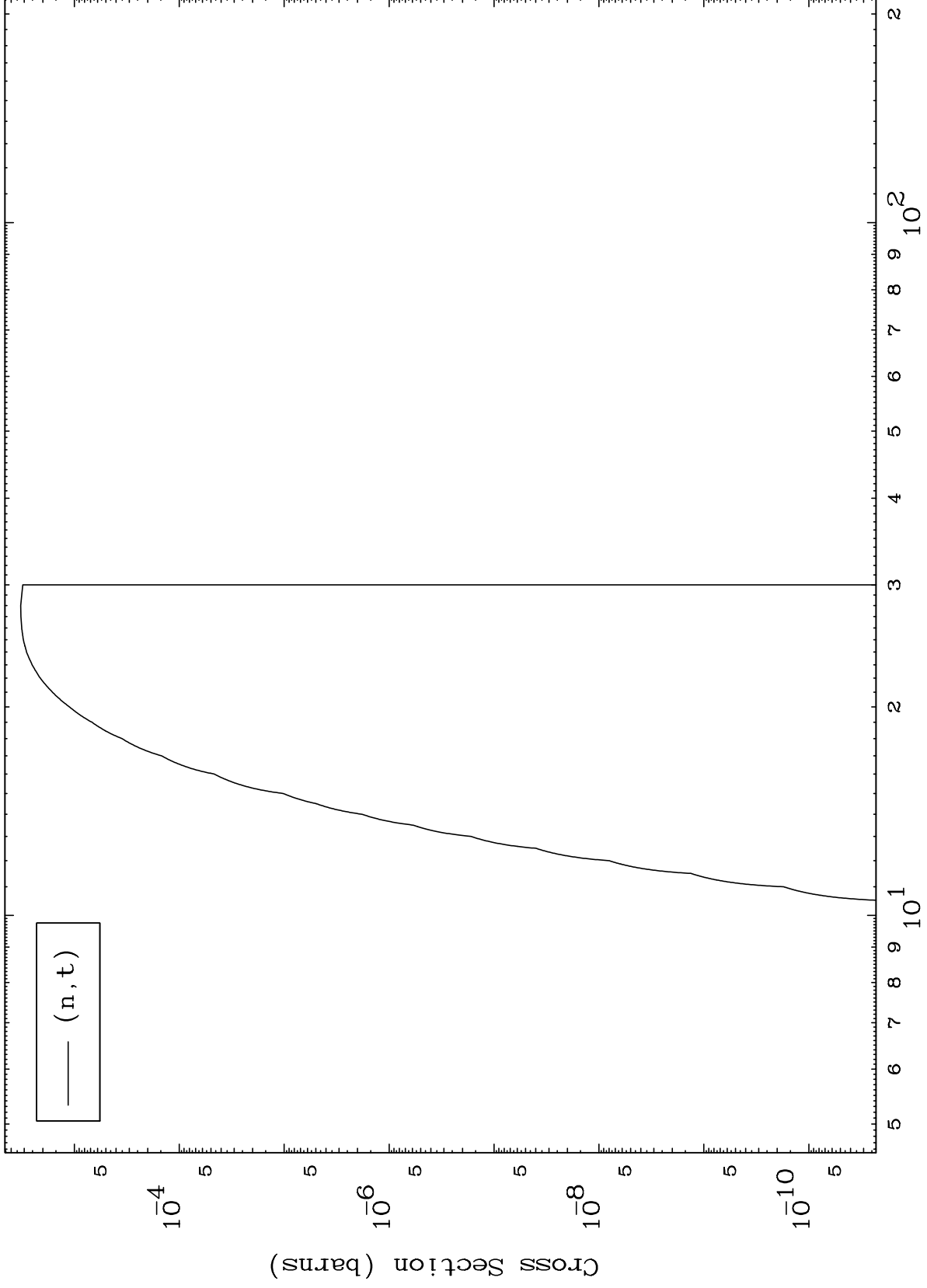
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92-U -236

MAT 9231

(n,t) Levels  
293 Kelvin Cross Sections

92-U -236



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Incident Energy (MeV)

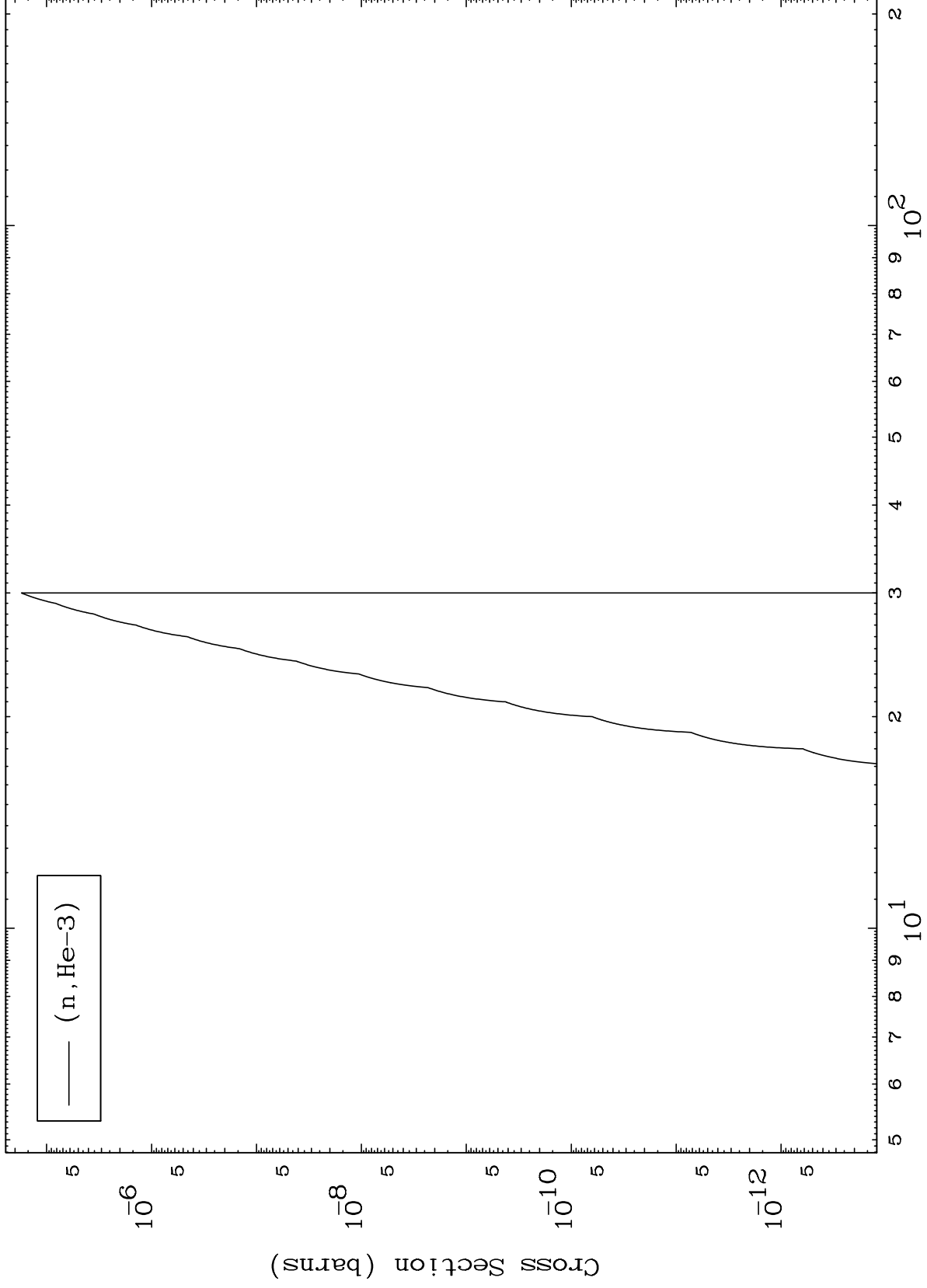
92-U -236



MAT 9231

(n,He3) Levels  
293 Kelvin Cross Sections

92-U -236



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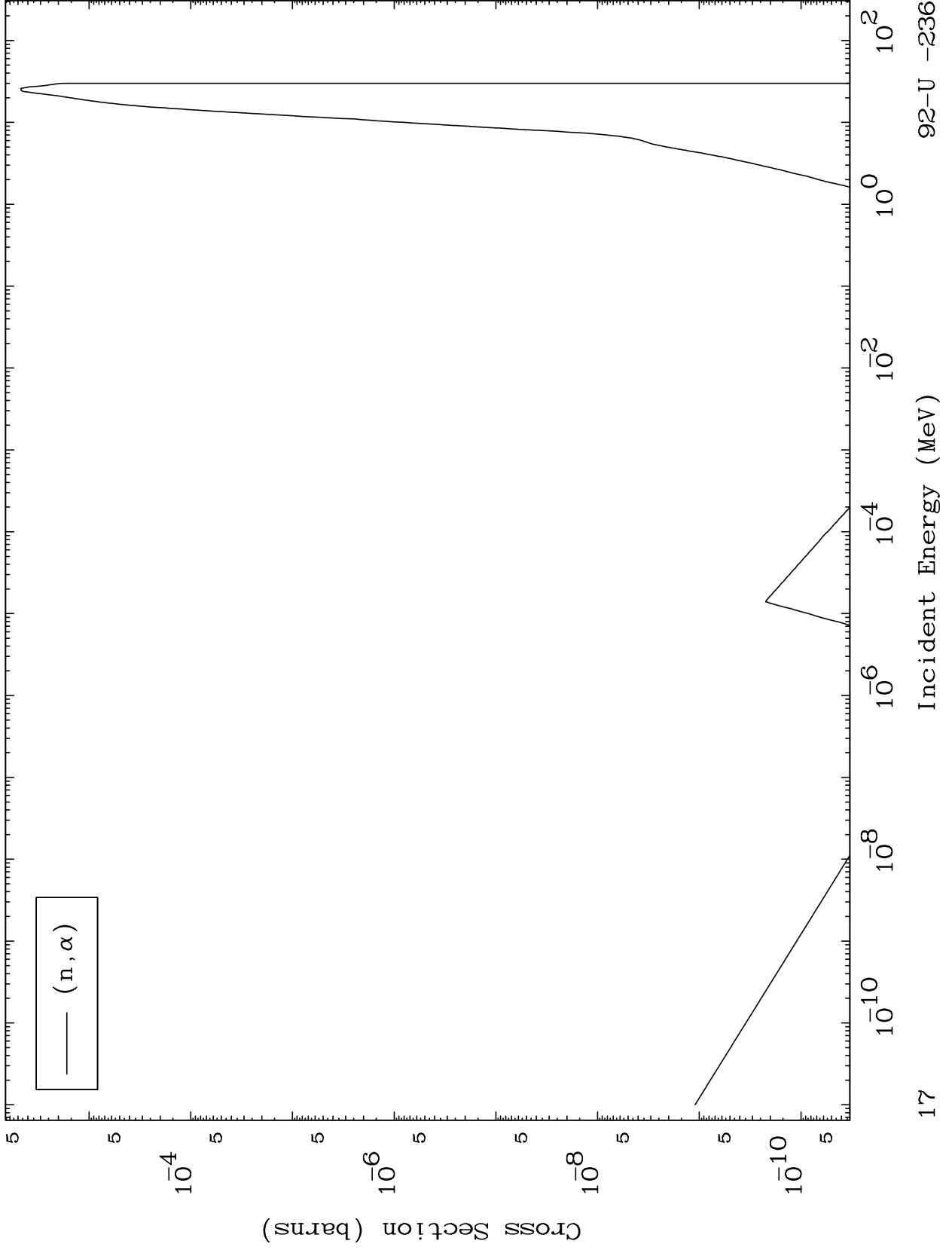
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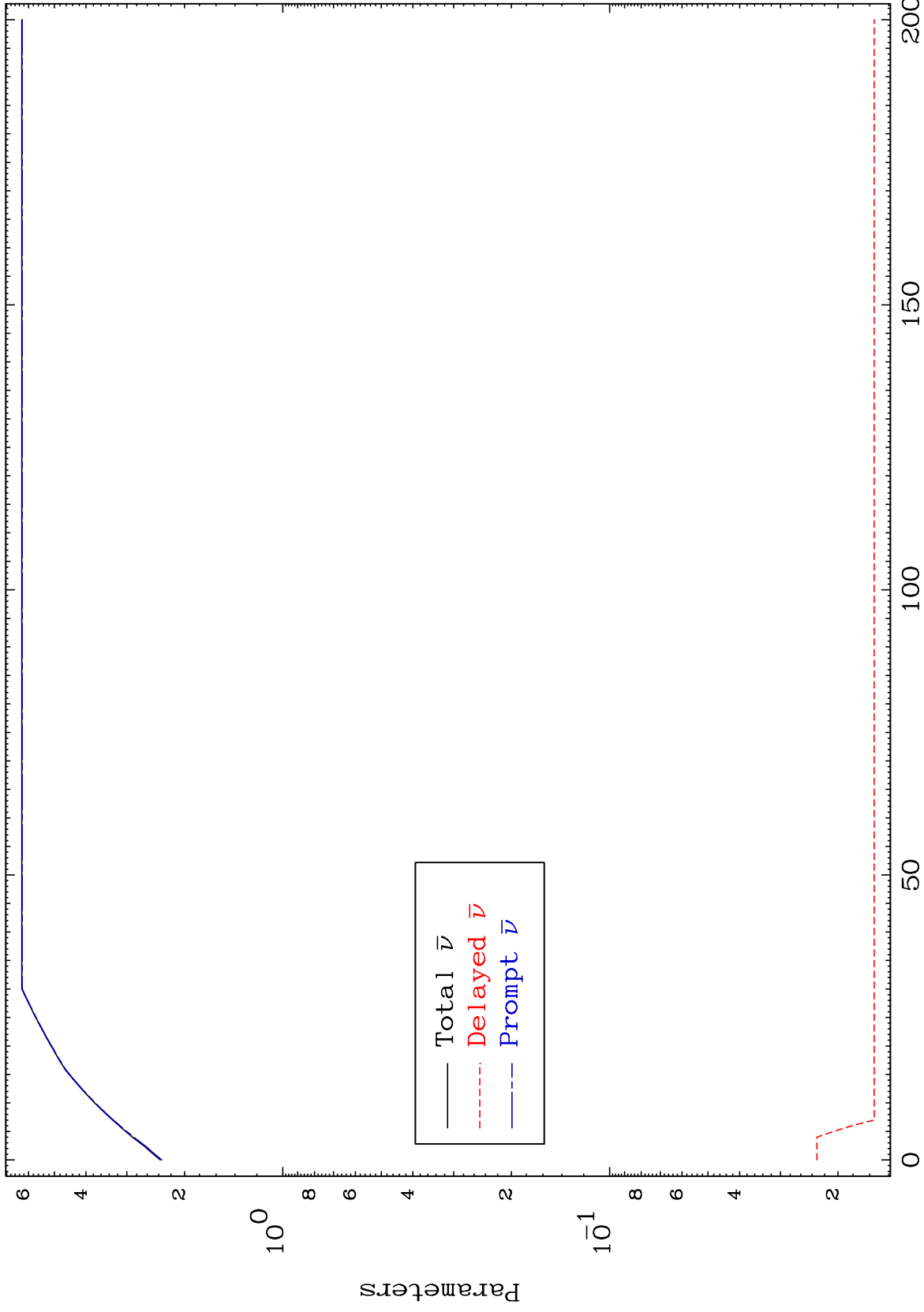
92-U -236

MAT 9231

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

92-U -236

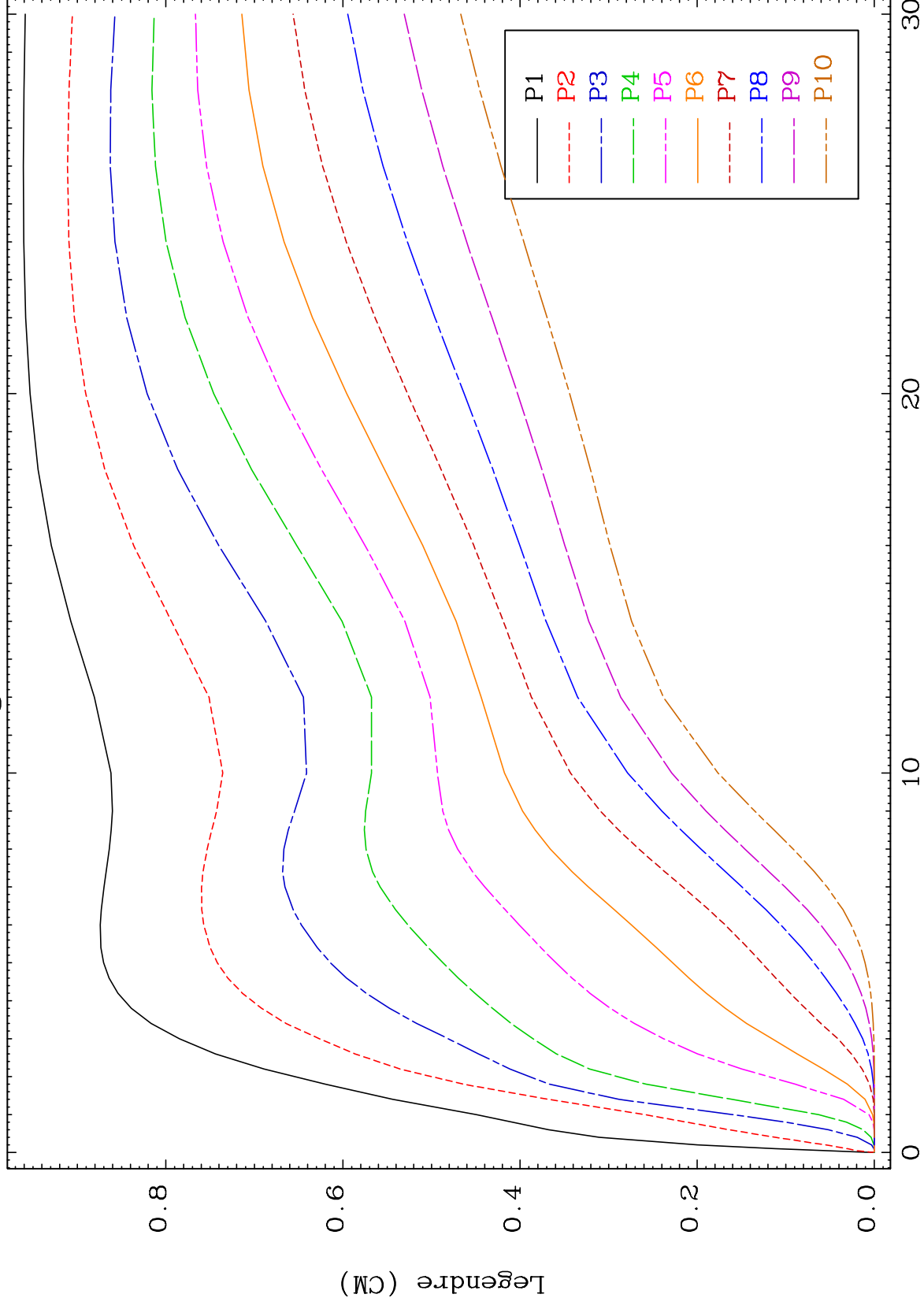




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Elastic Legendre Coefficients

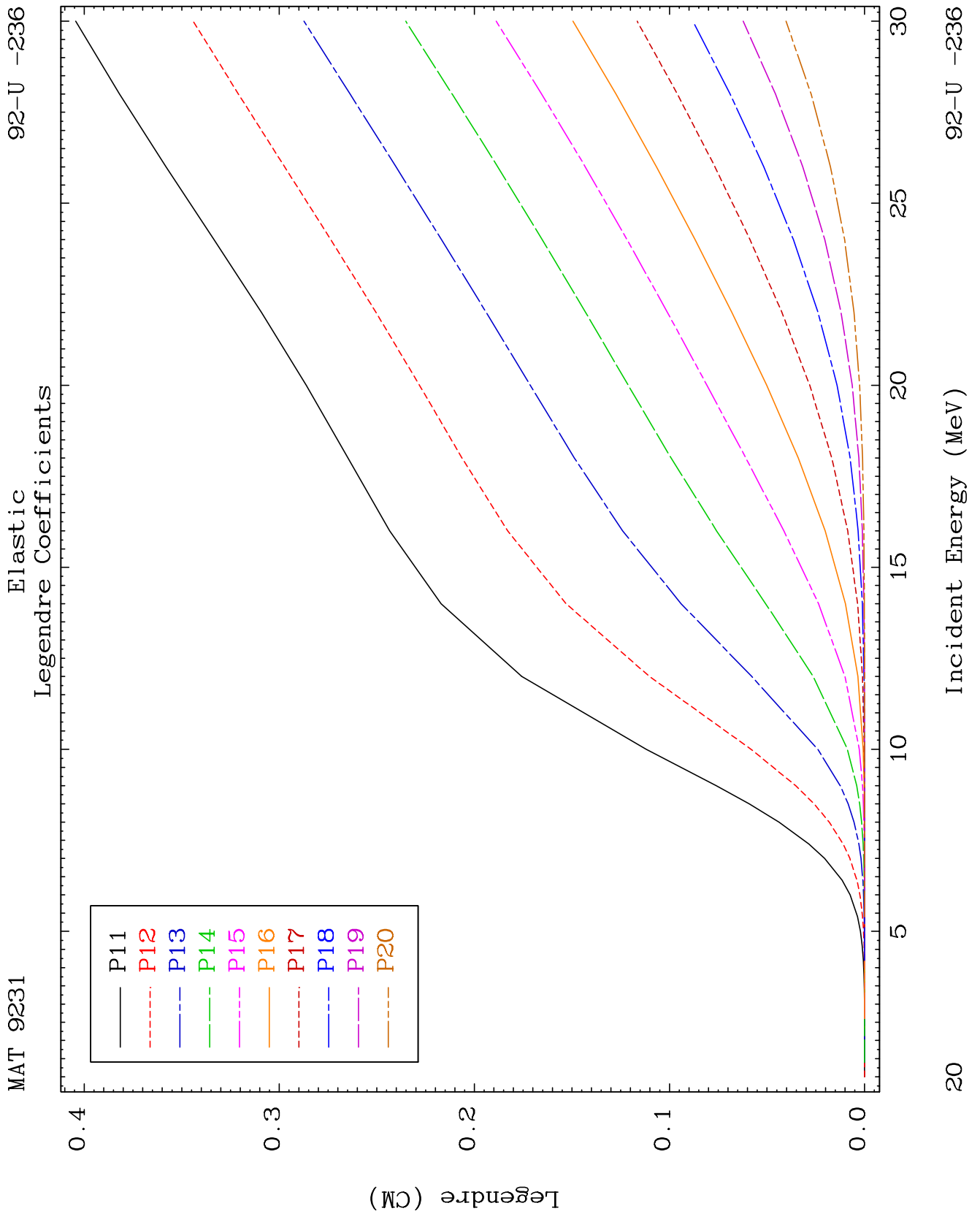
92-U -236



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Incident Energy (MeV)

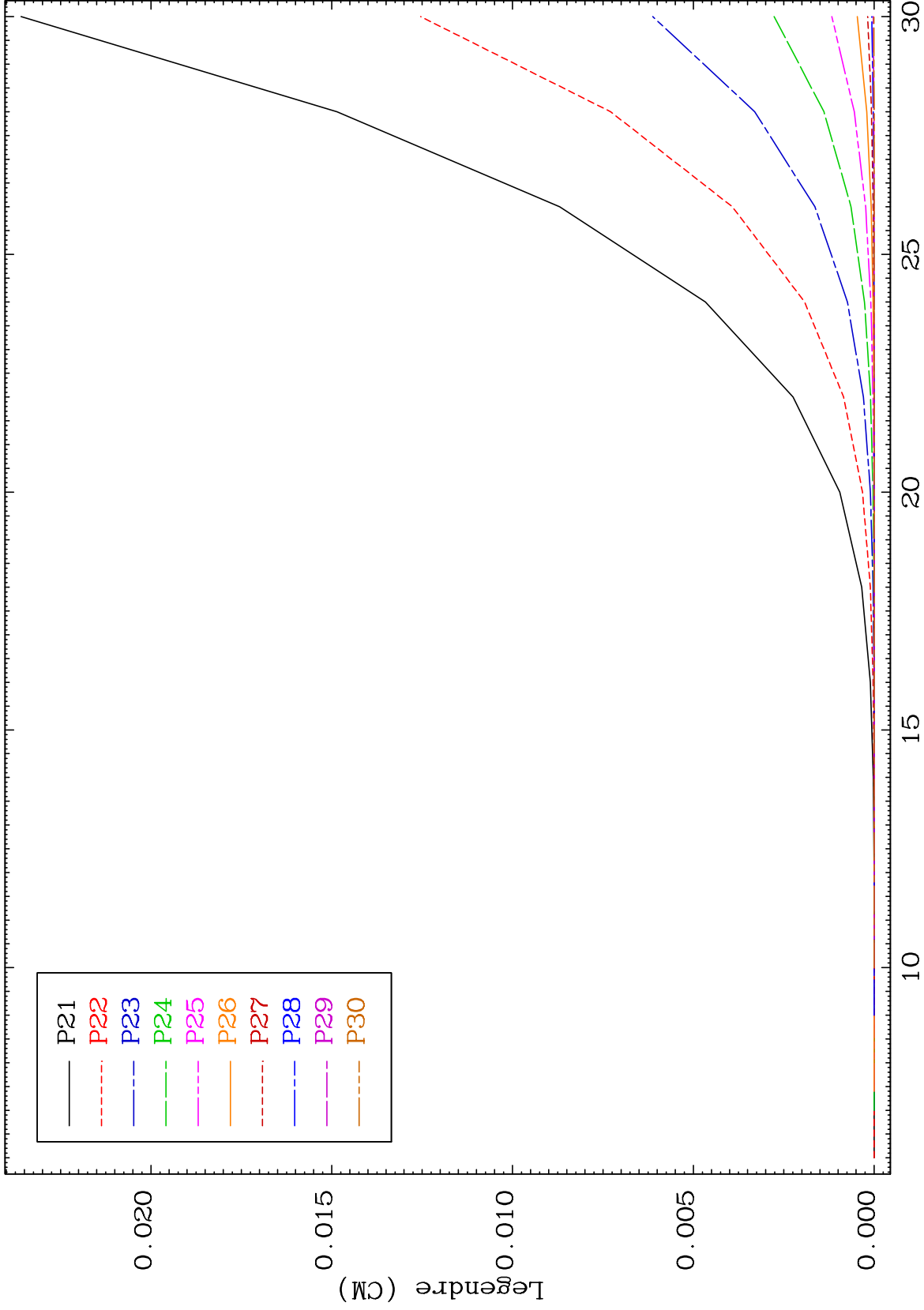
92-U -236



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Elastic Legendre Coefficients

92-U -236



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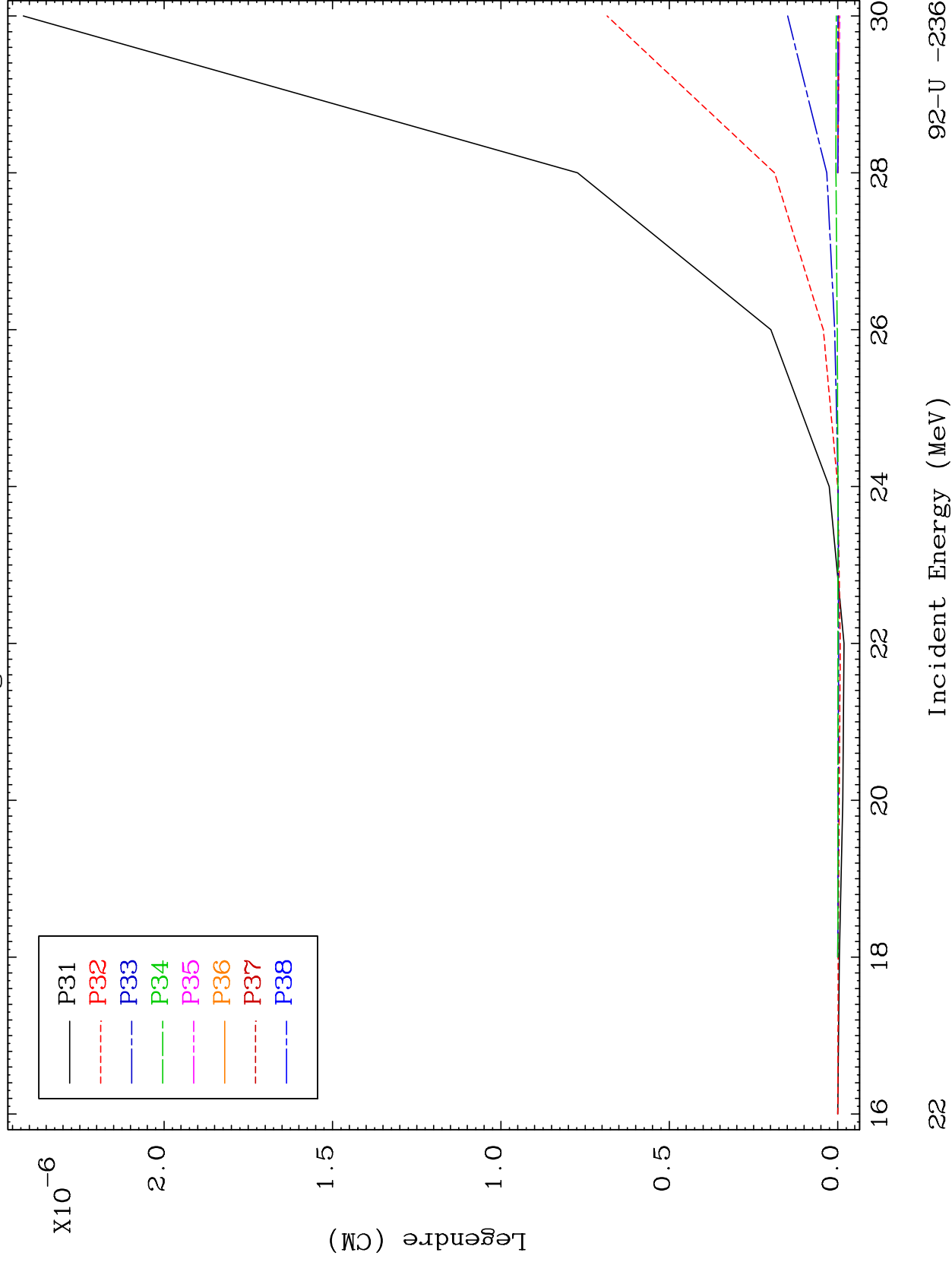
Incident Energy (MeV)

92-U -236

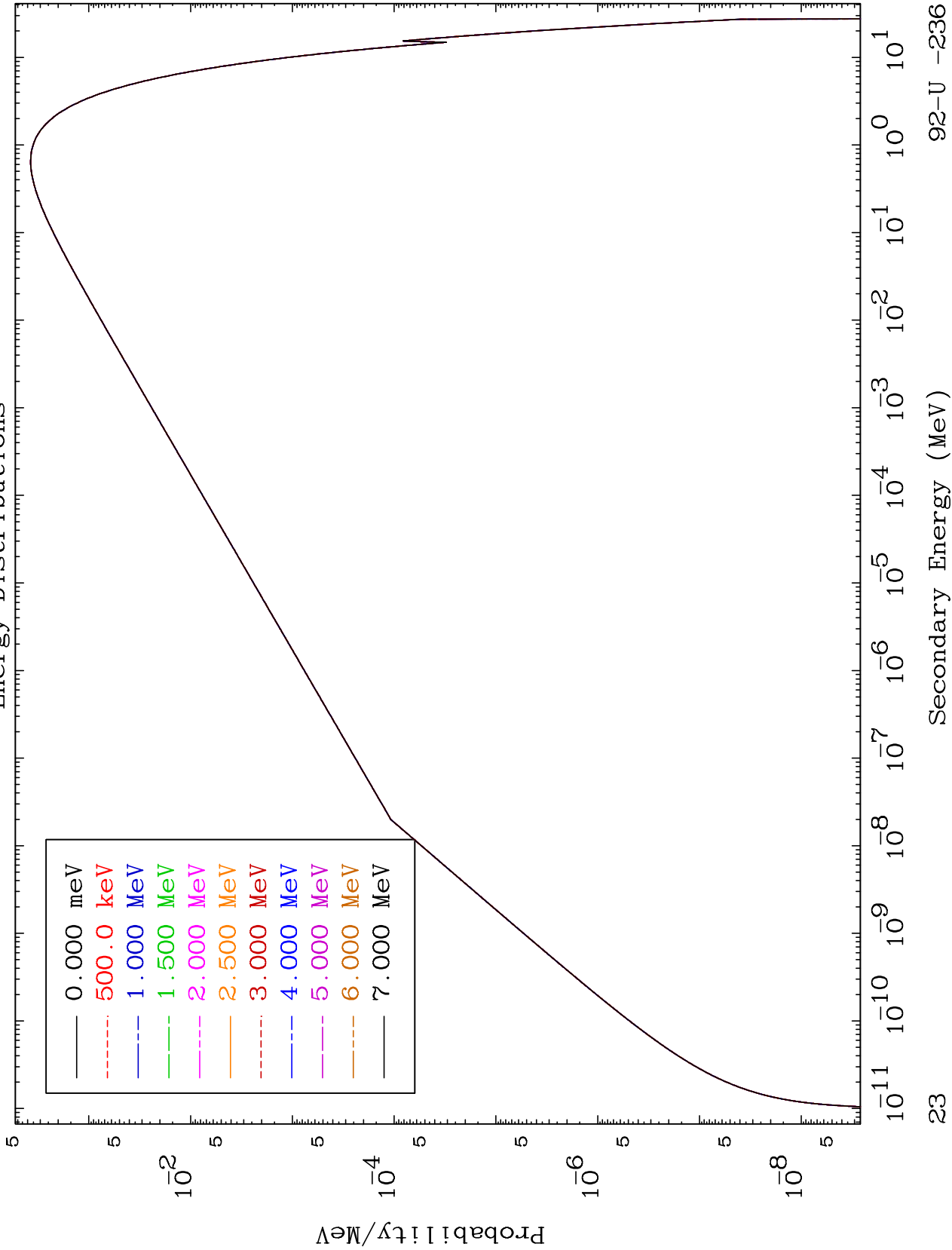
MAT 9231

Elastic  
Legendre Coefficients

92-U -236

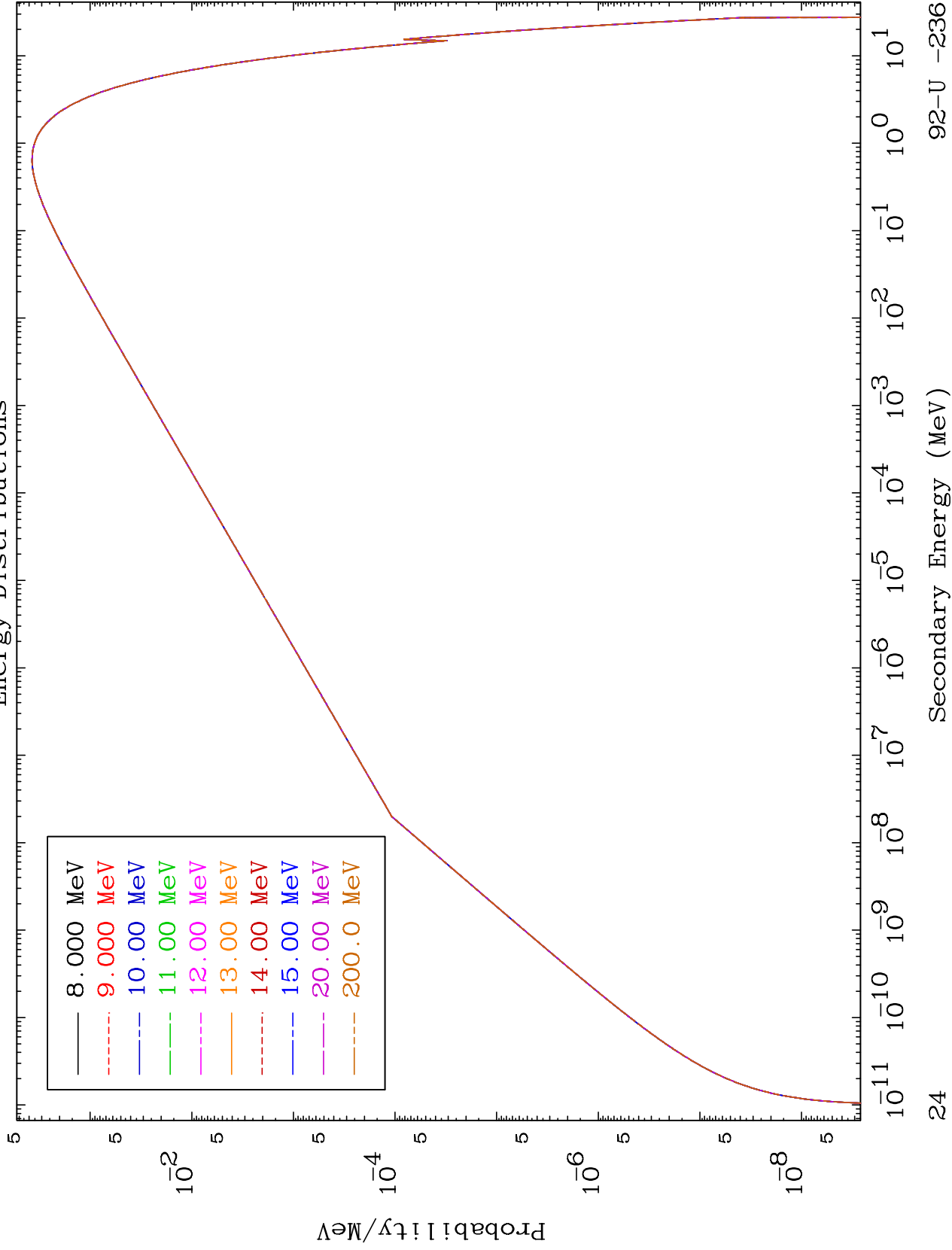


### Fission Energy Distributions





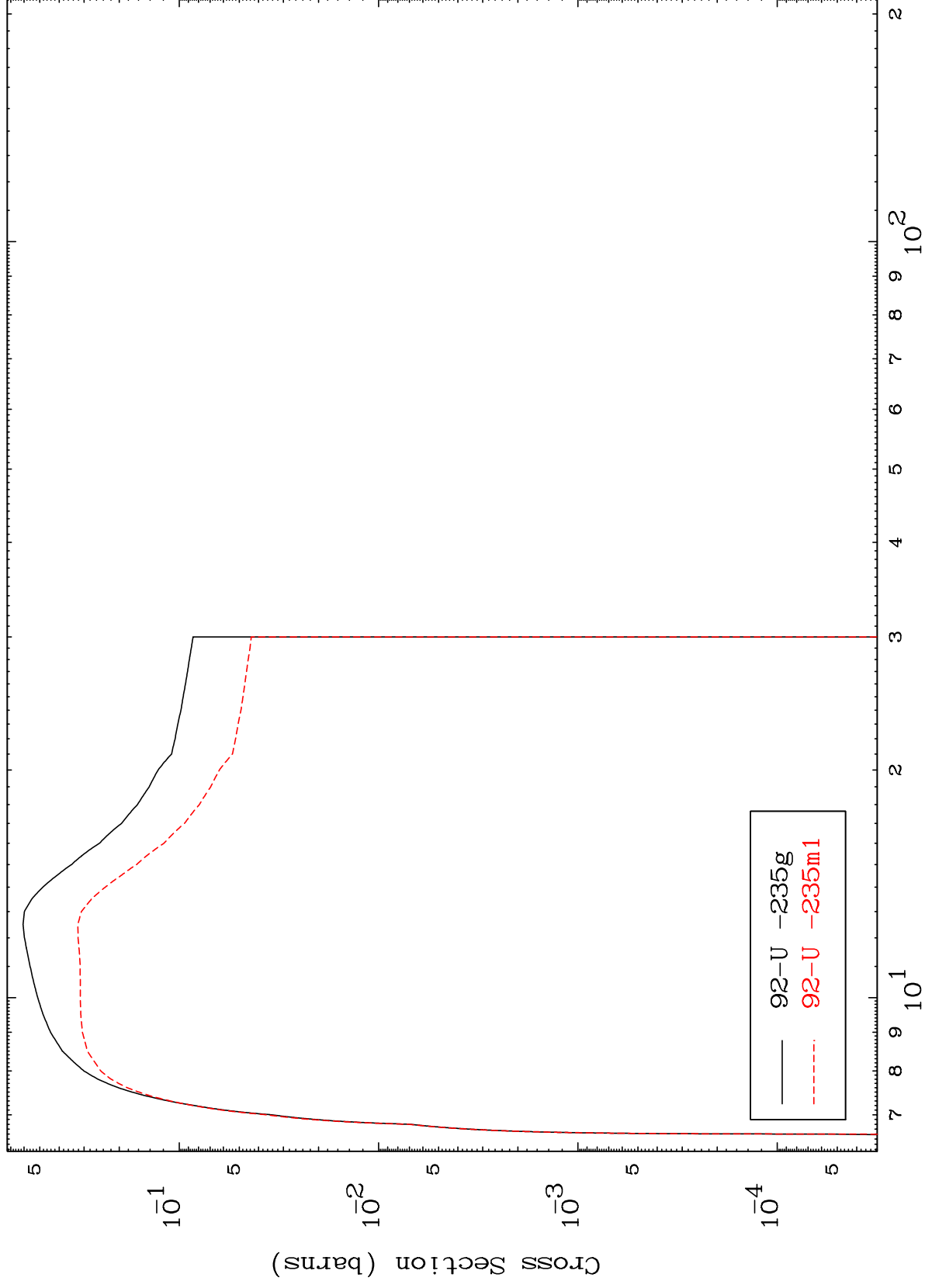
Fission Energy Distributions



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(n,2n)  
Radionuclide Production Cross Section



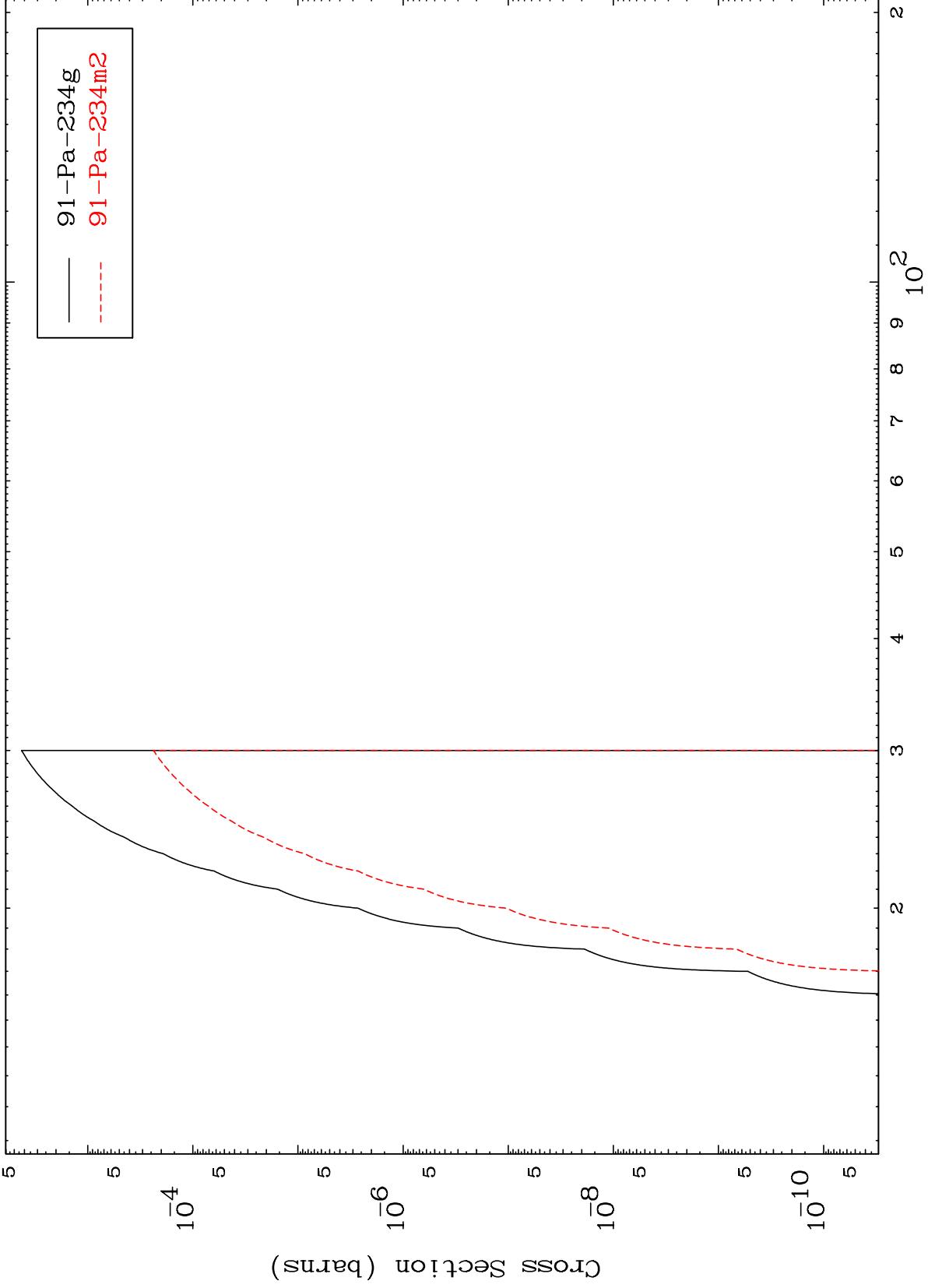
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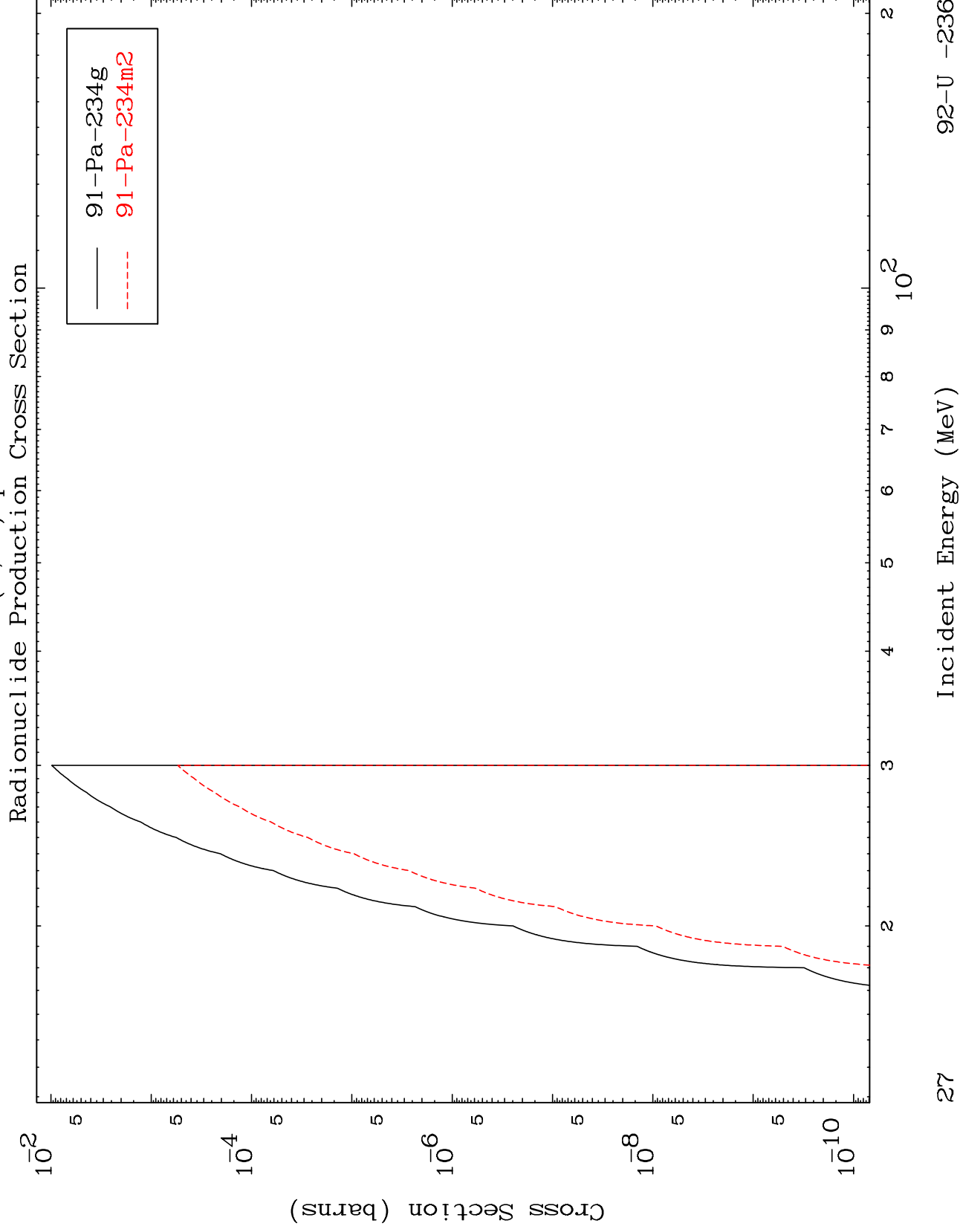
Incident Energy (MeV)

92-U -236

(n,n') d

Radionuclide Production Cross Section





(n,t)  
Radionuclide Production Cross Section

