

Program EVALPLOT  
(Version 2021-1)

by

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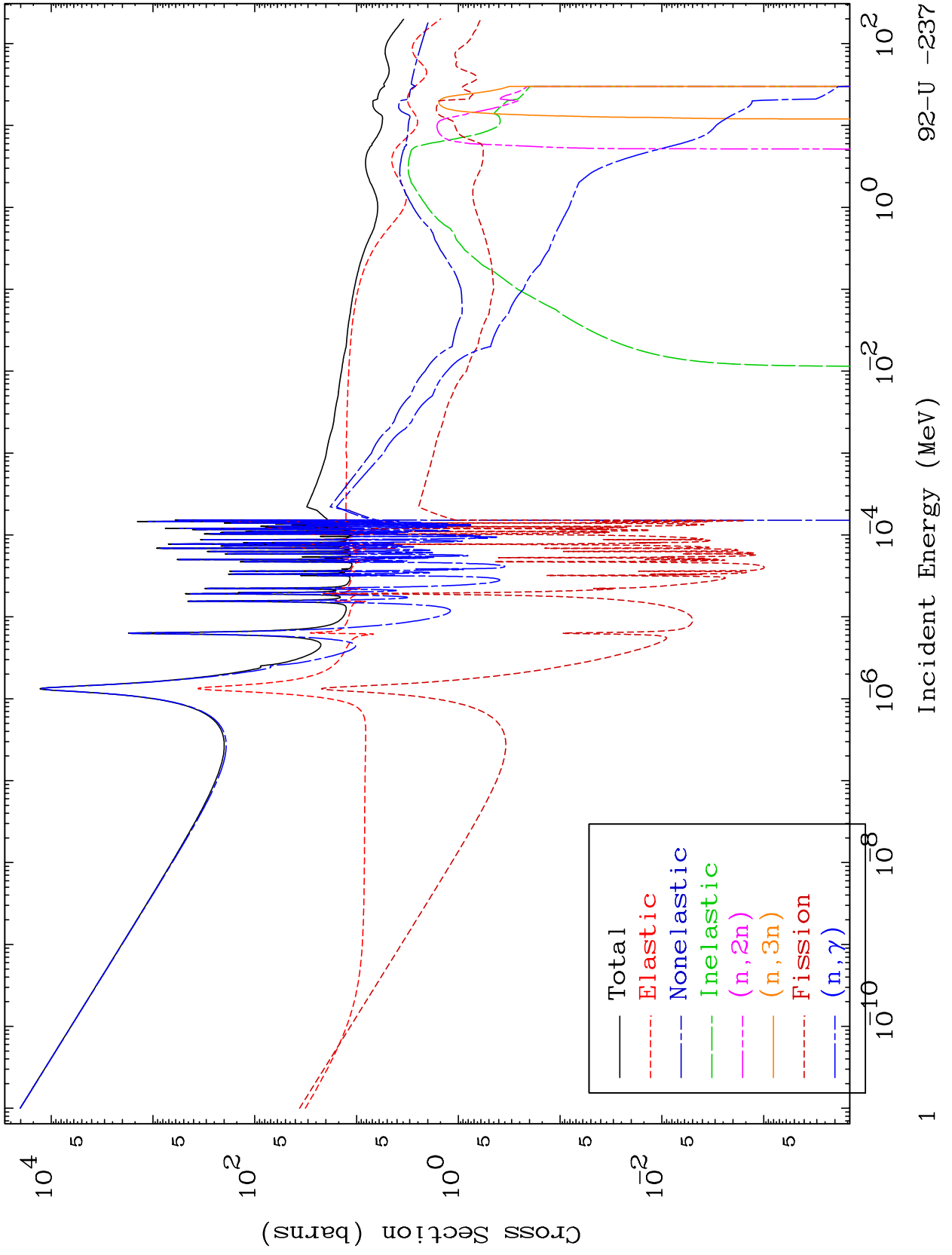
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9234

Neutron Major  
293 Kelvin Cross Sections

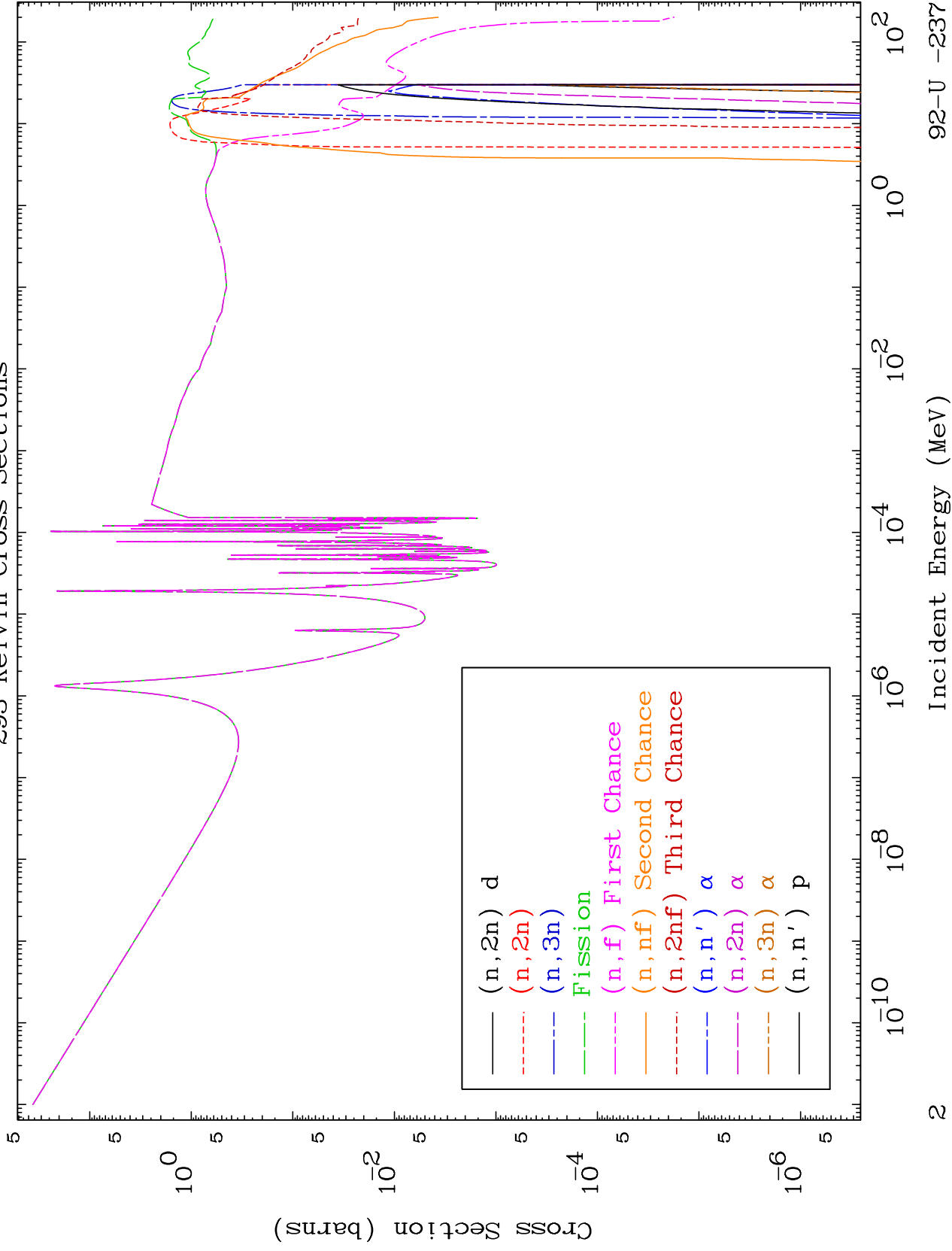
92-U -237



MAT 9234

Neutron Absorption  
293 Kelvin Cross Sections

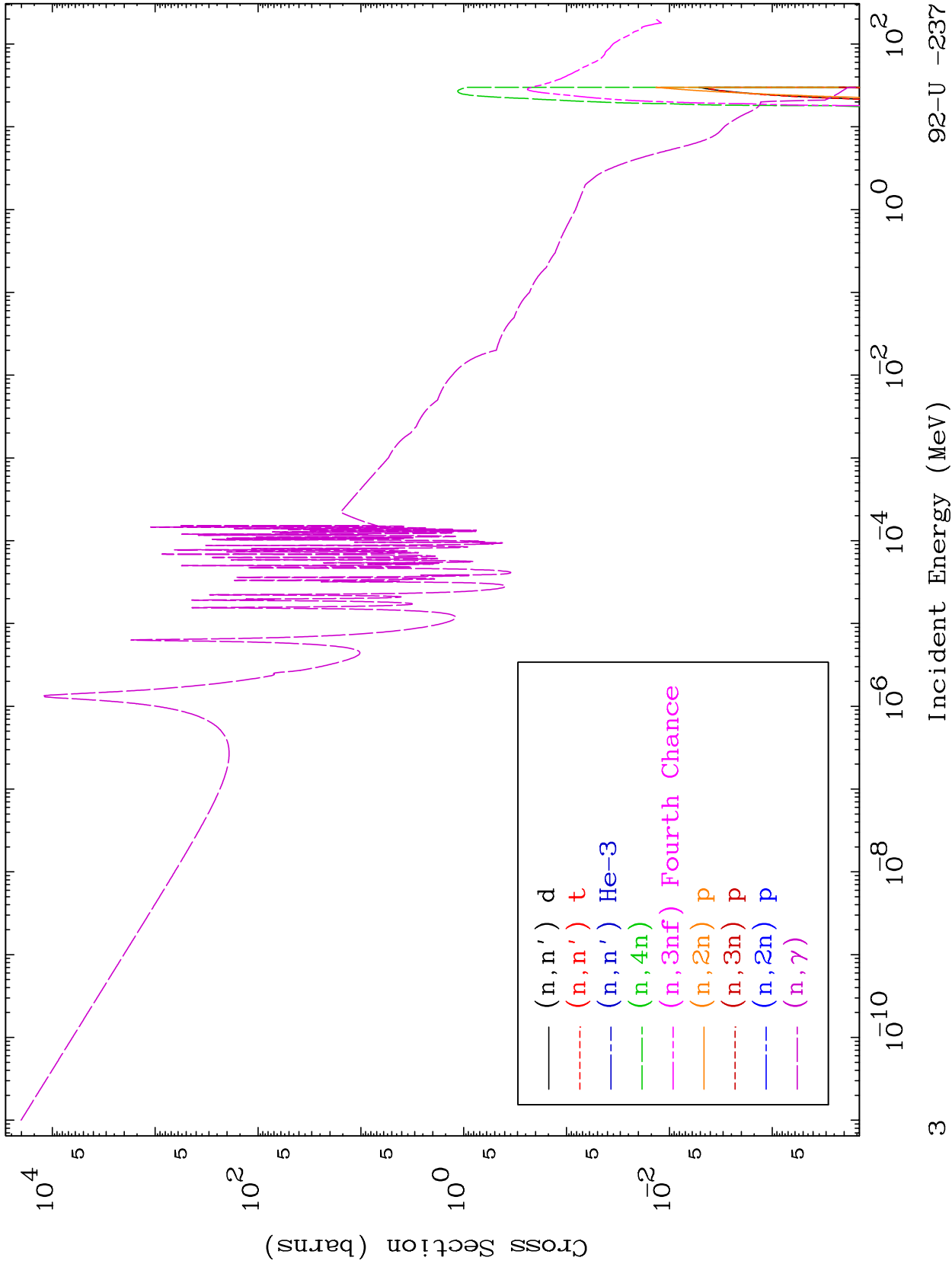
92-U -237



MAT 9234

Neutron Absorption  
293 Kelvin Cross Sections

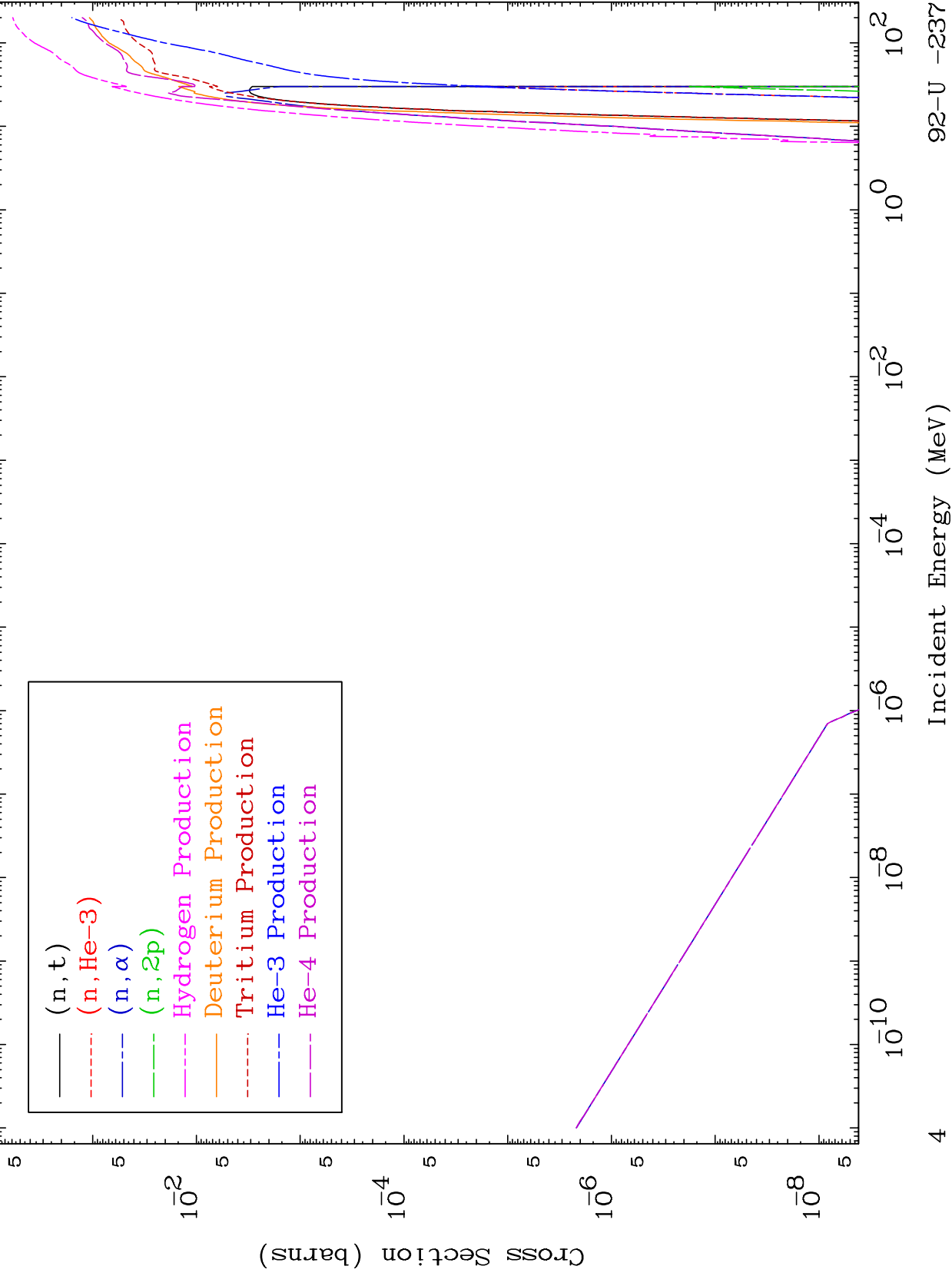
92-U -237



MAT 9234

Neutron Absorption  
293 Kelvin Cross Sections

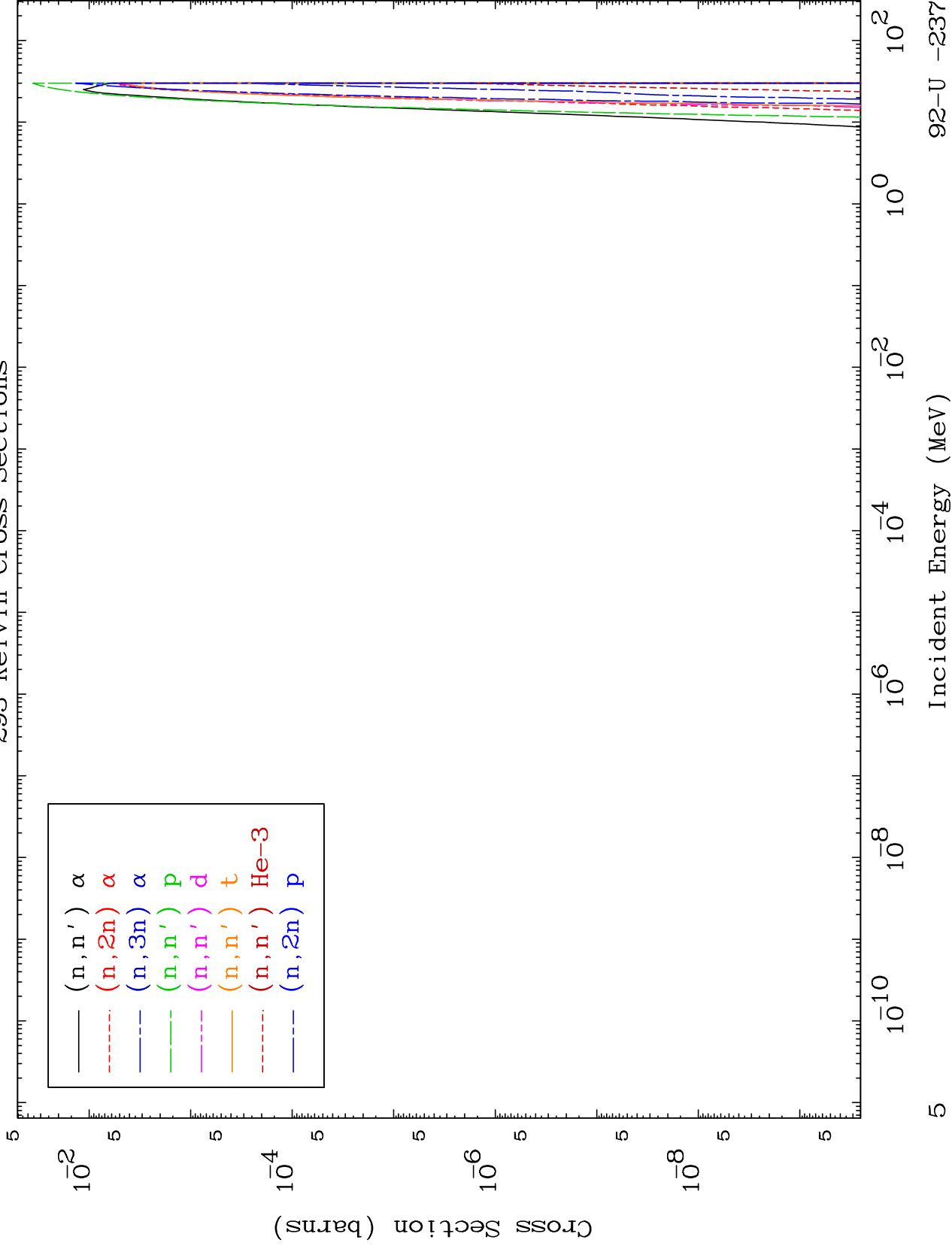
92-U -237



MAT 9234

Charged Particle  
293 Kelvin Cross Sections

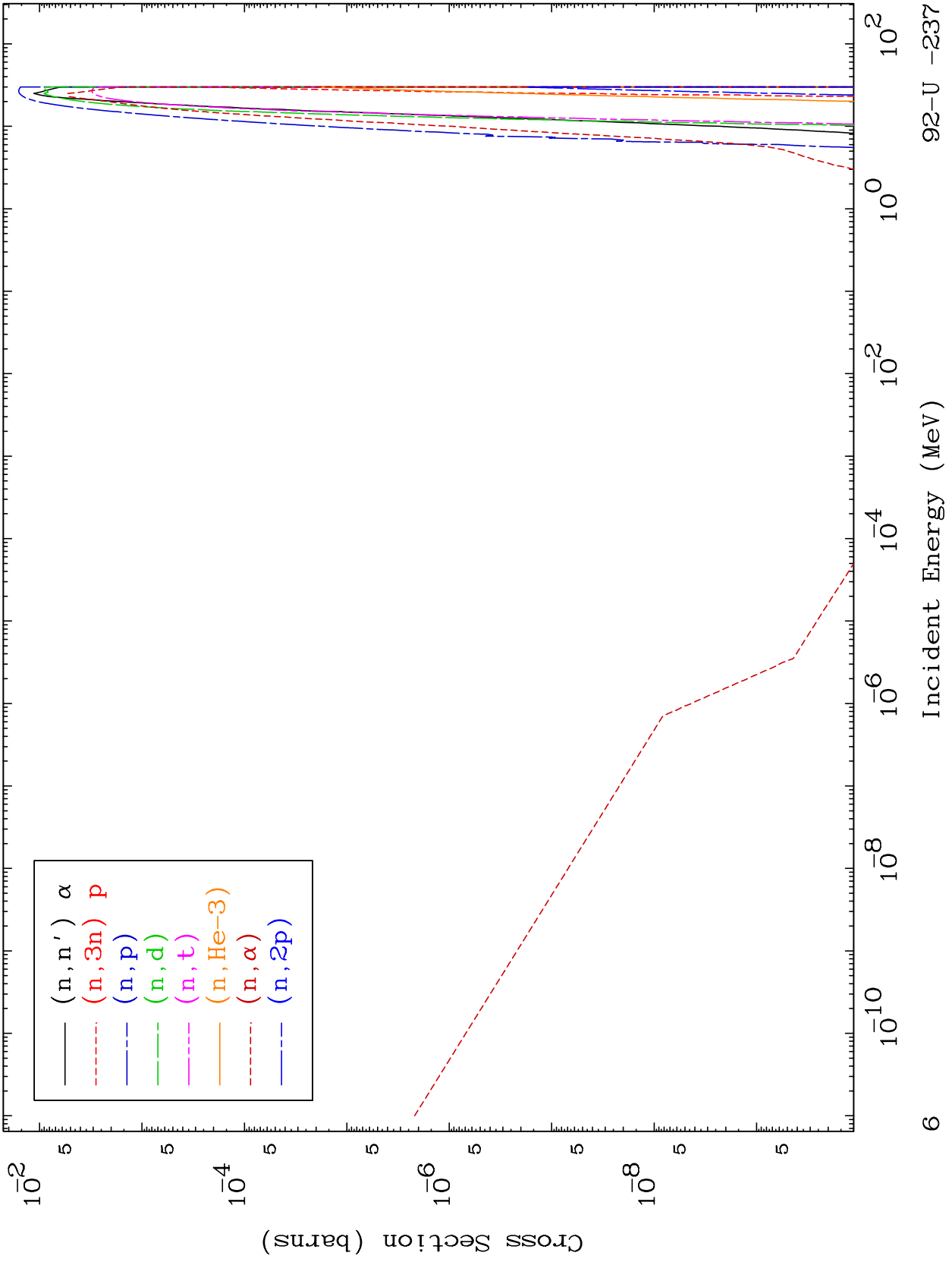
92-U -237



MAT 9234

Charged Particle  
293 Kelvin Cross Sections

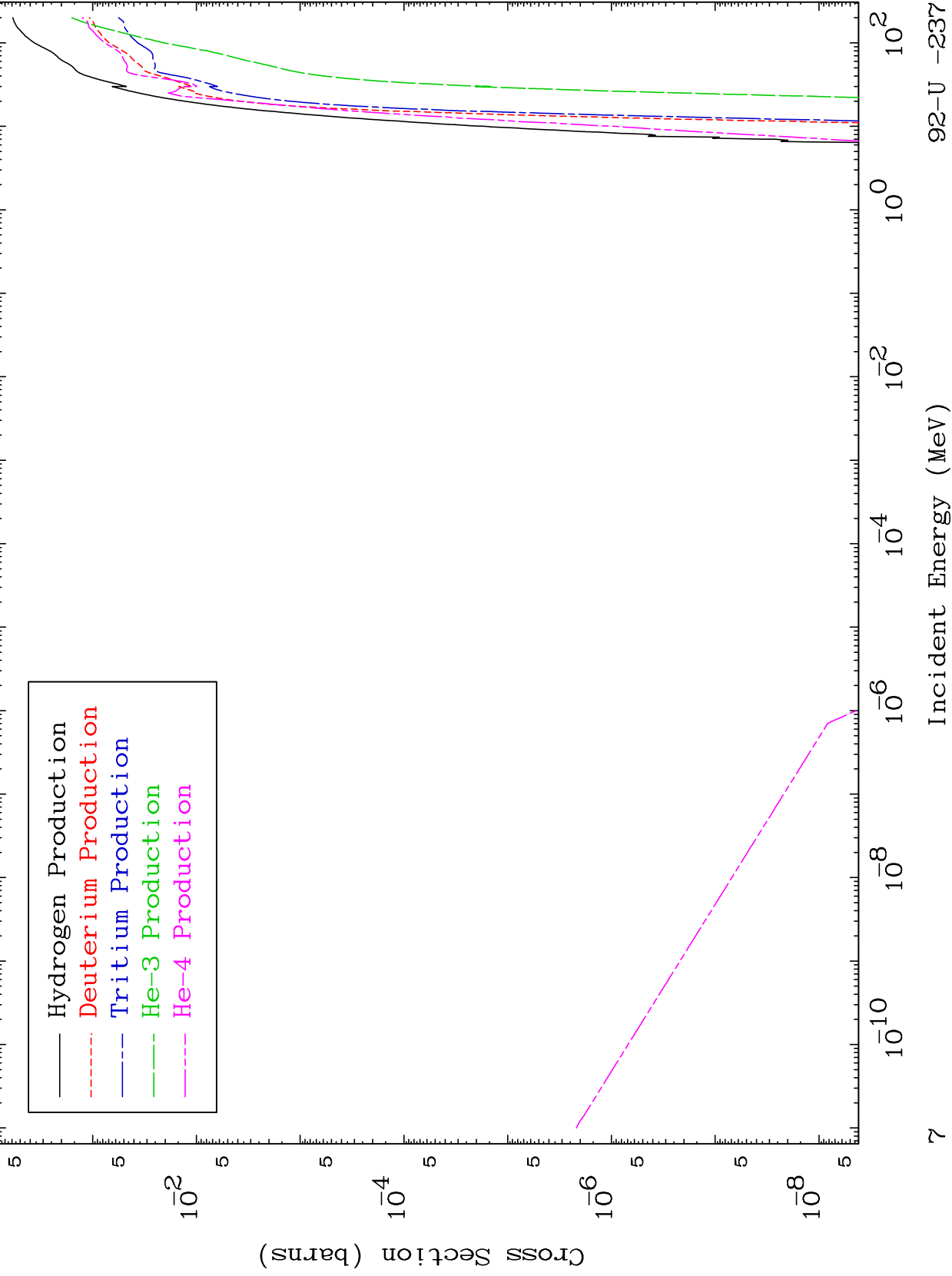
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MAT 9234

Particle Production  
293 Kelvin Cross Sections

92-U -237

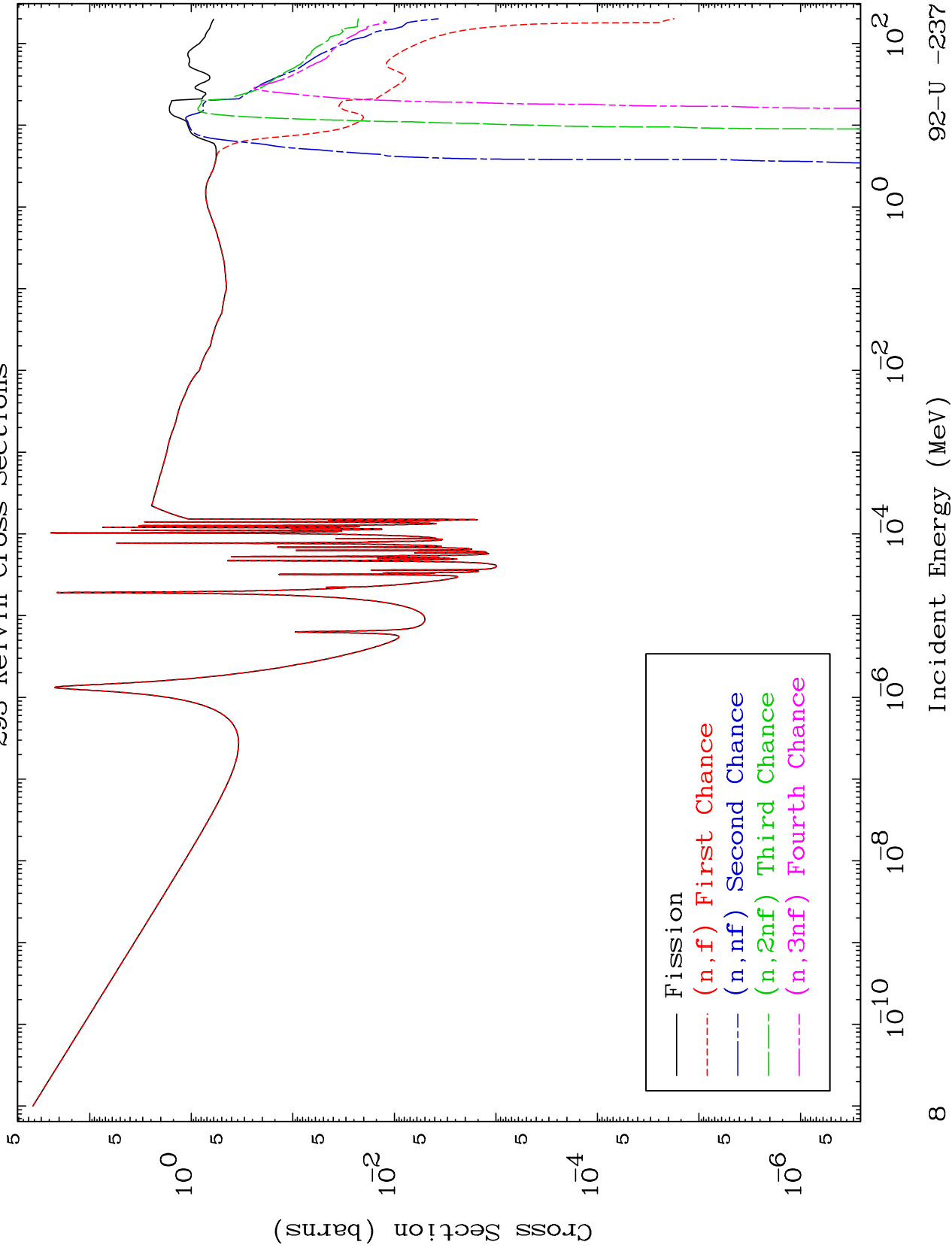


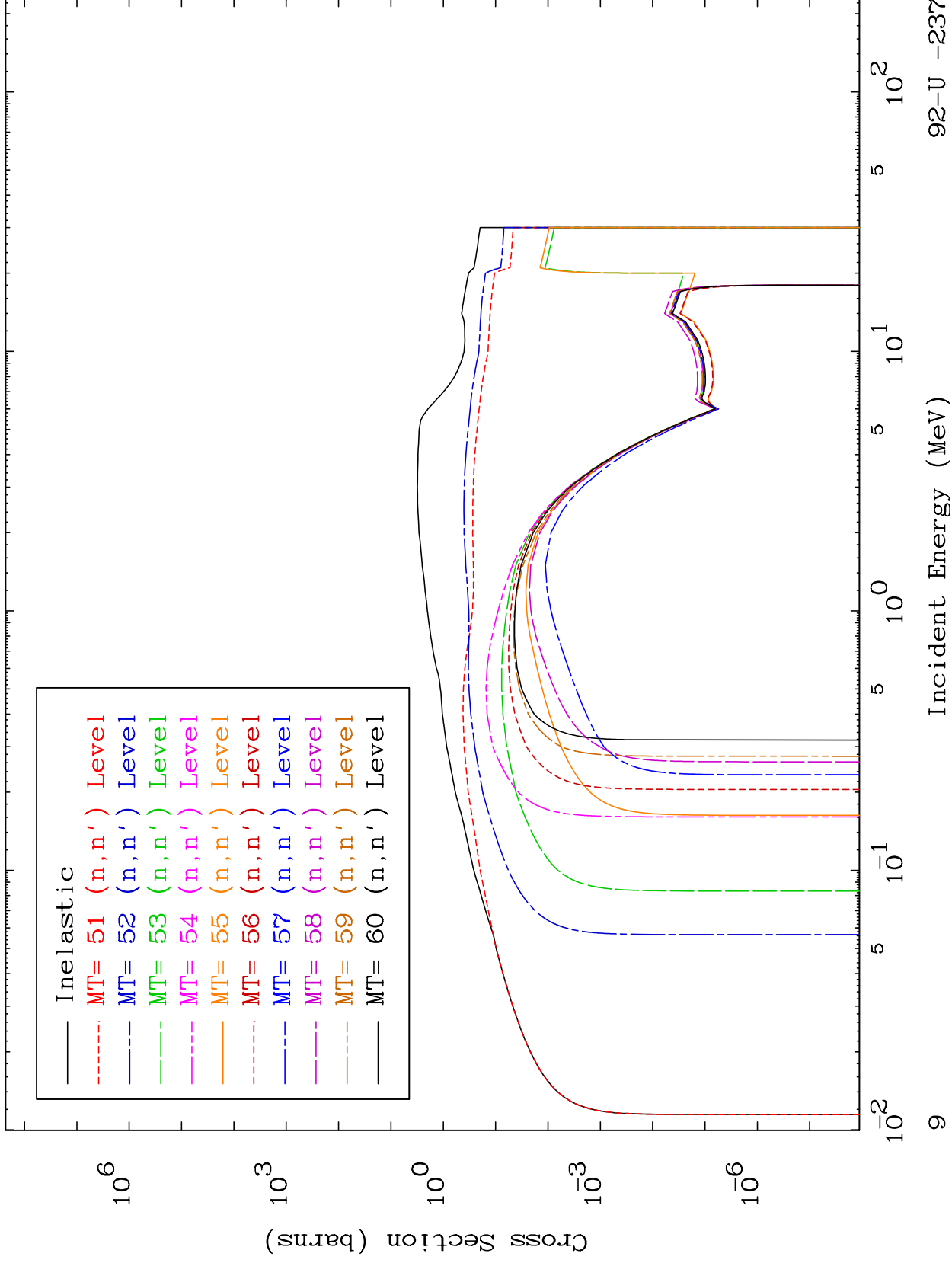


MAT 9234

Fission  
293 Kelvin Cross Sections

92-U -237

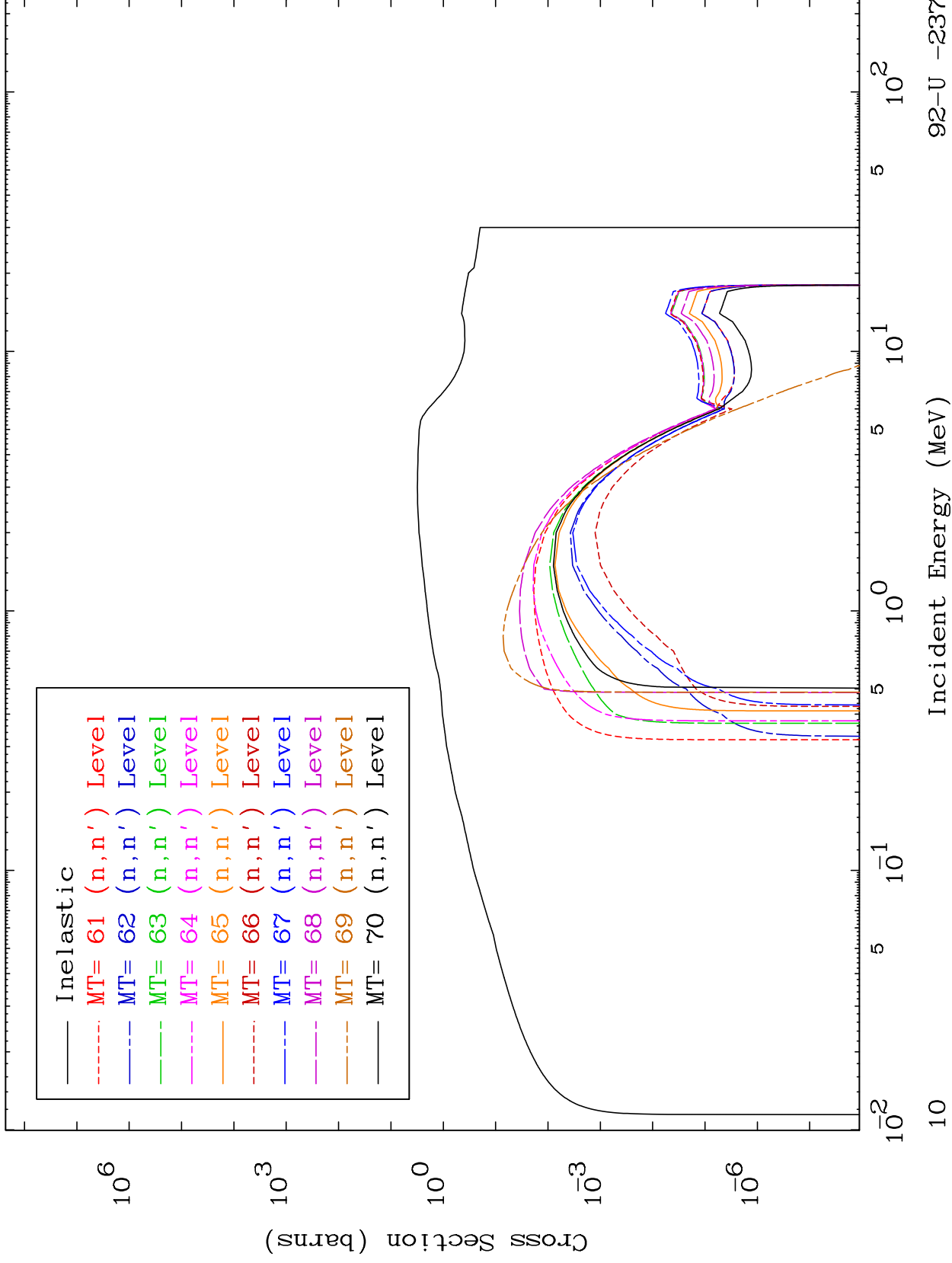




MAT 9234

(n,n') Levels  
293 Kelvin Cross Sections

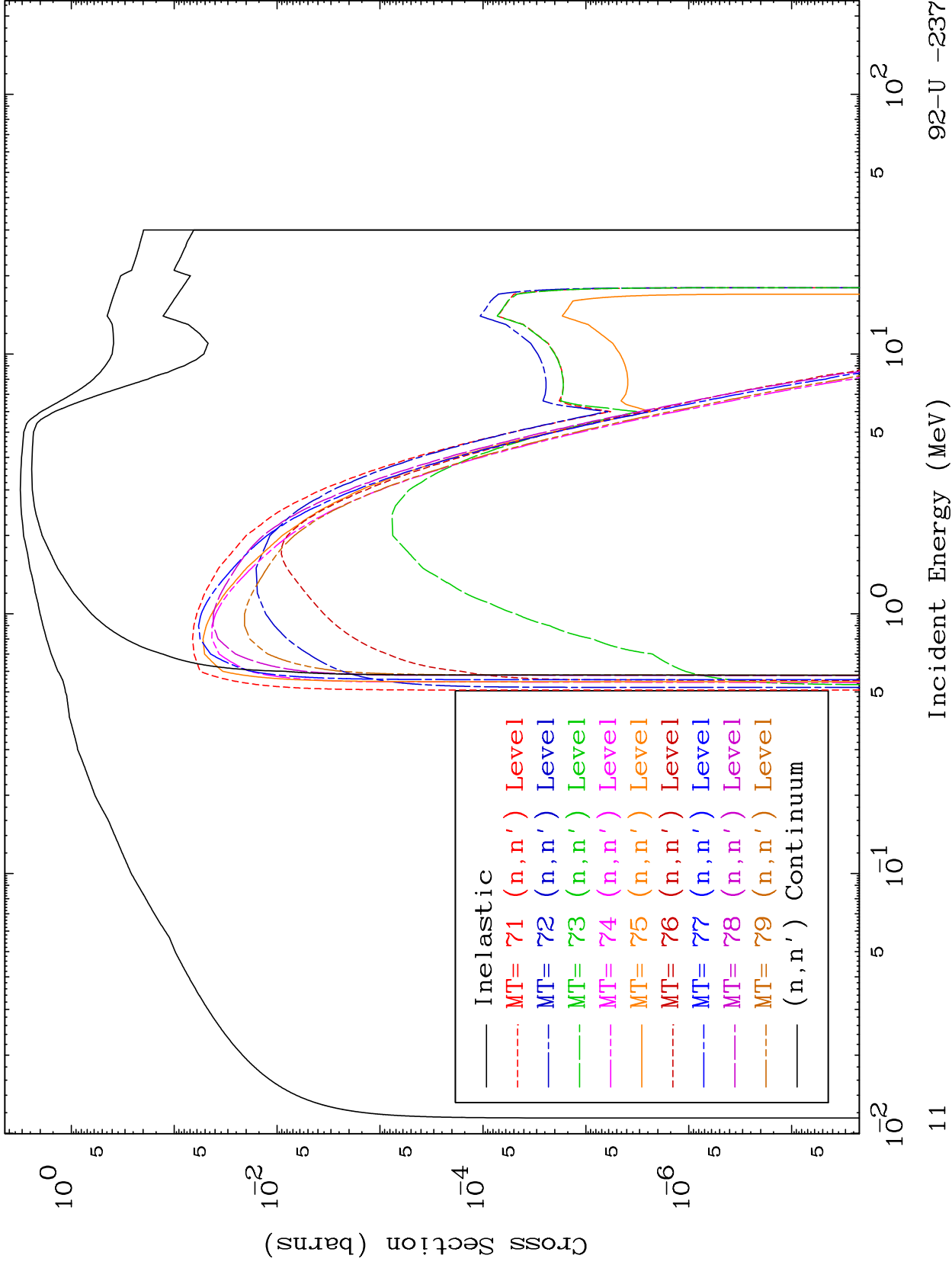
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MAT 9234

(n,n') Levels  
293 Kelvin Cross Sections

92-U -237



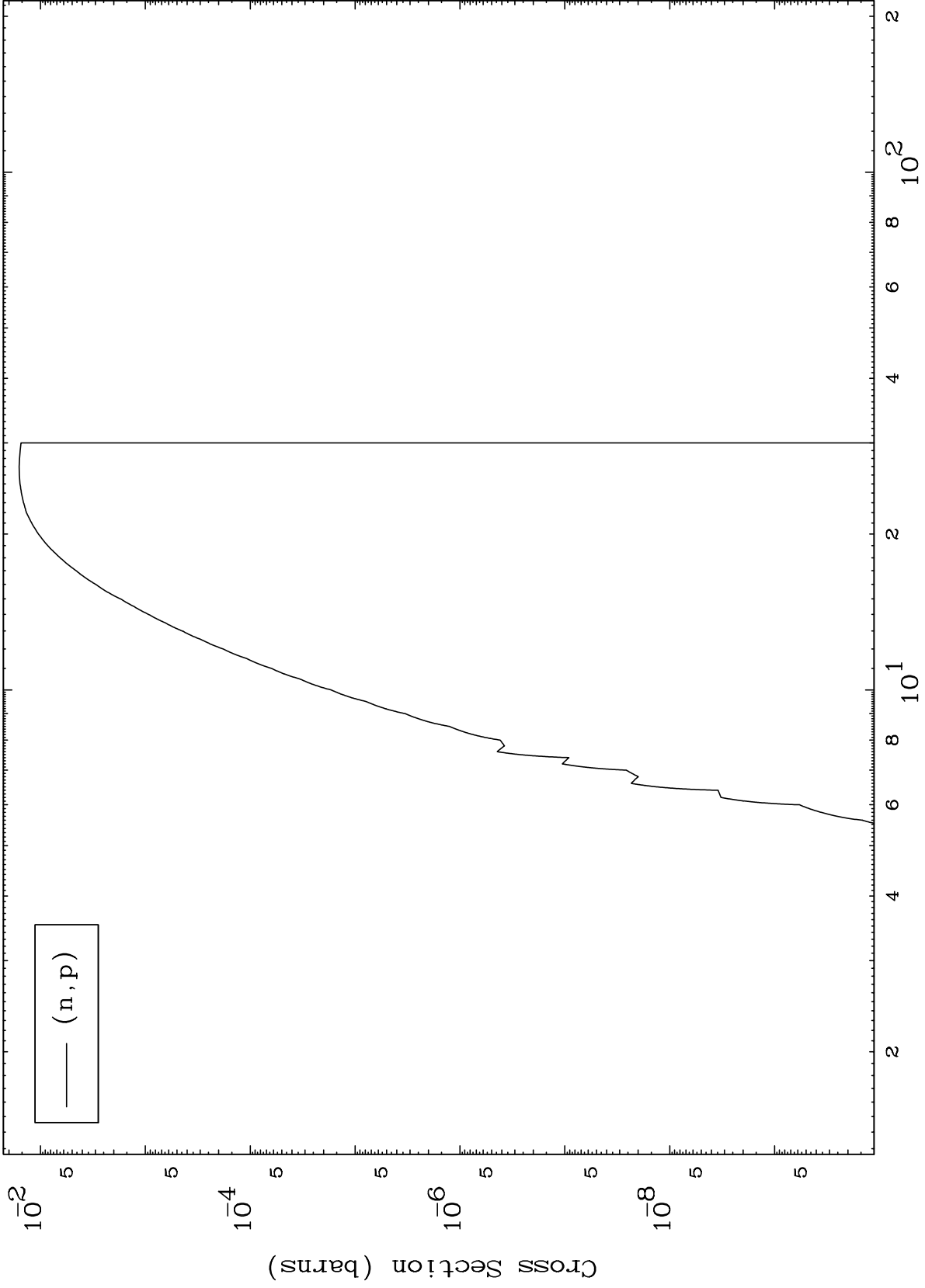
11

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MAT 9234

(n,p) Levels  
293 Kelvin Cross Sections

92-U -237



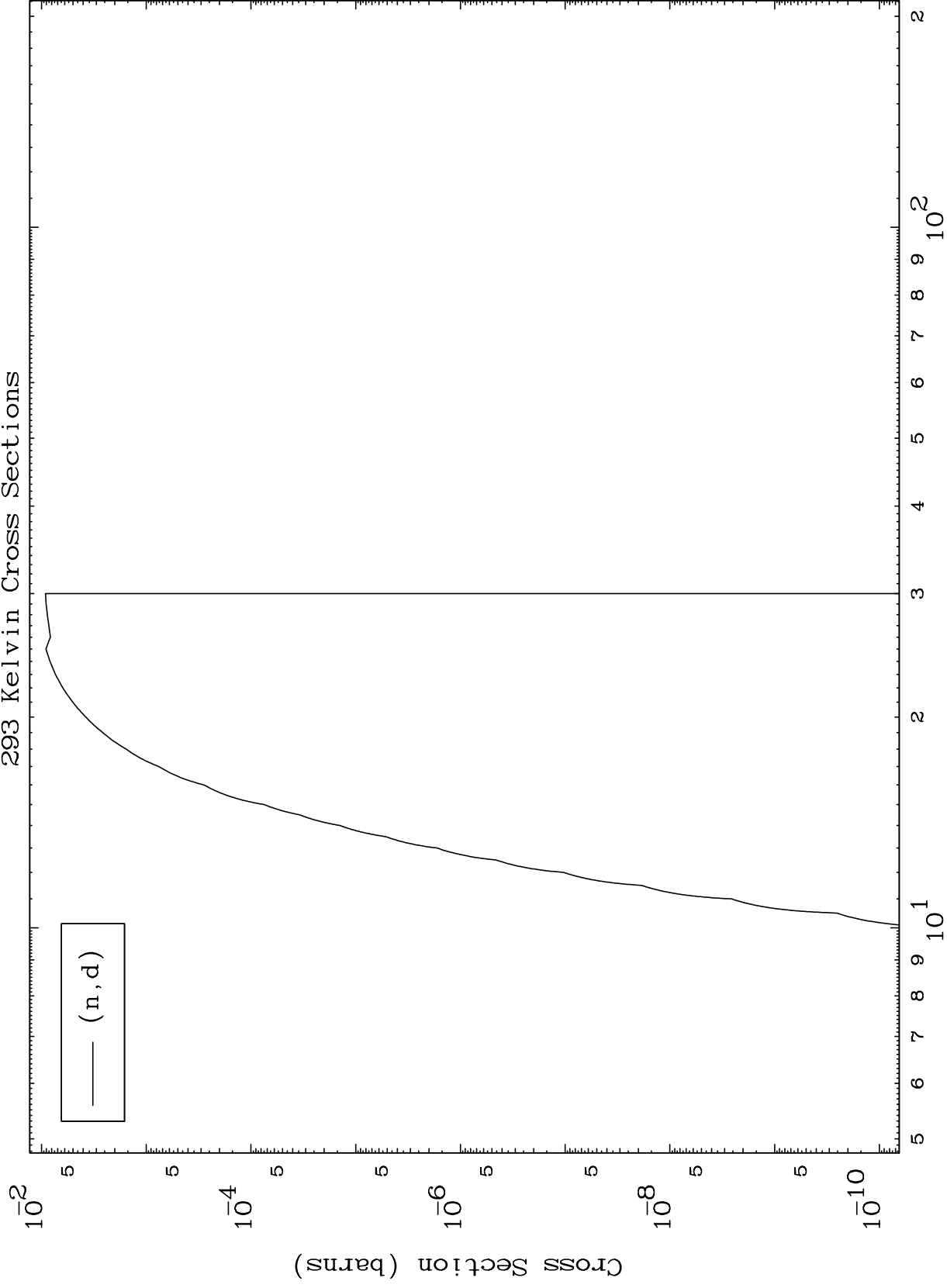
12

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MAT 9234

(n,d) Levels  
293 Kelvin Cross Sections

92-U -237



13

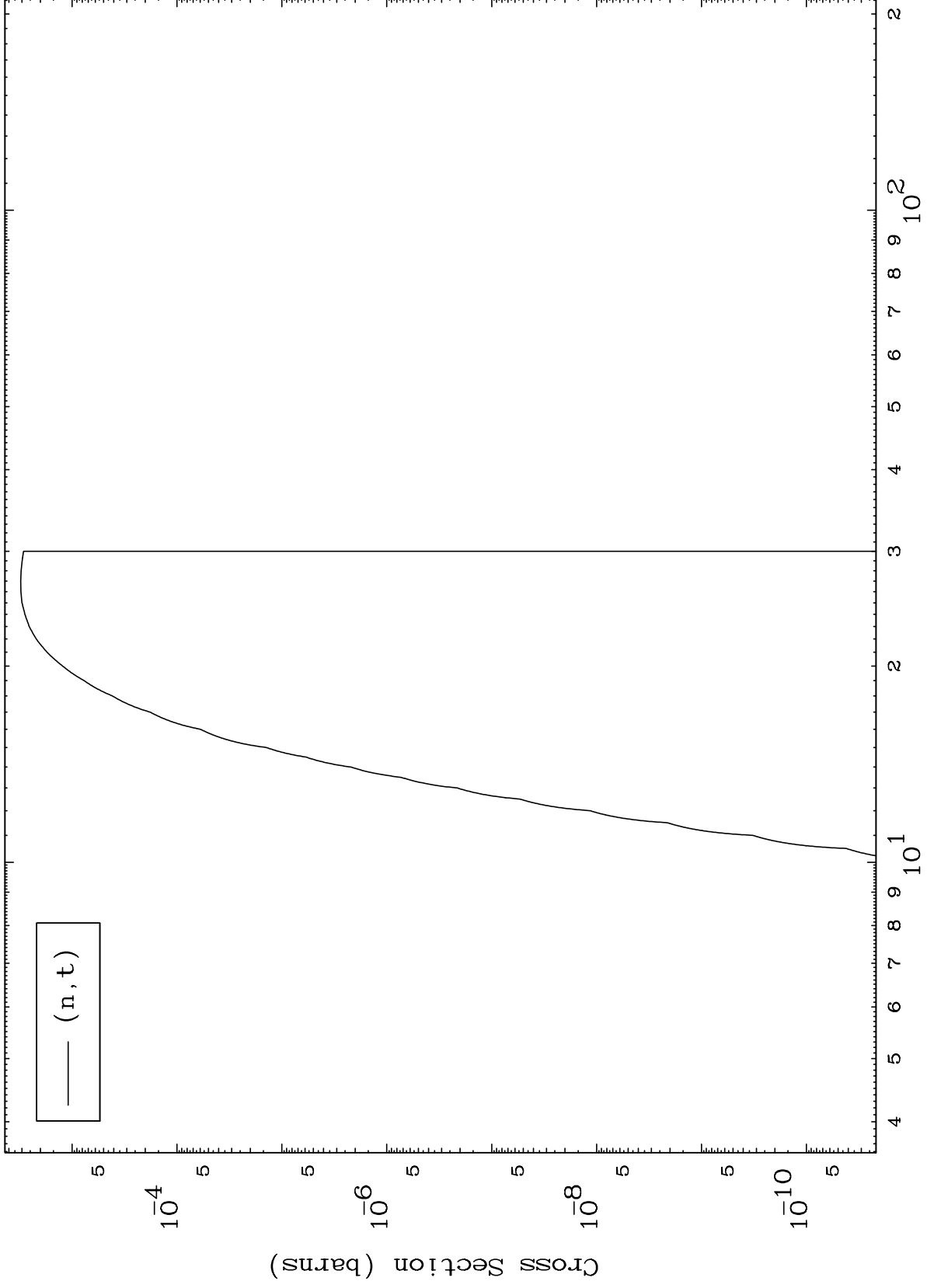
Incident Energy (MeV)

92-U -237

MAT 9234

(n,t) Levels  
293 Kelvin Cross Sections

92-U -237



14

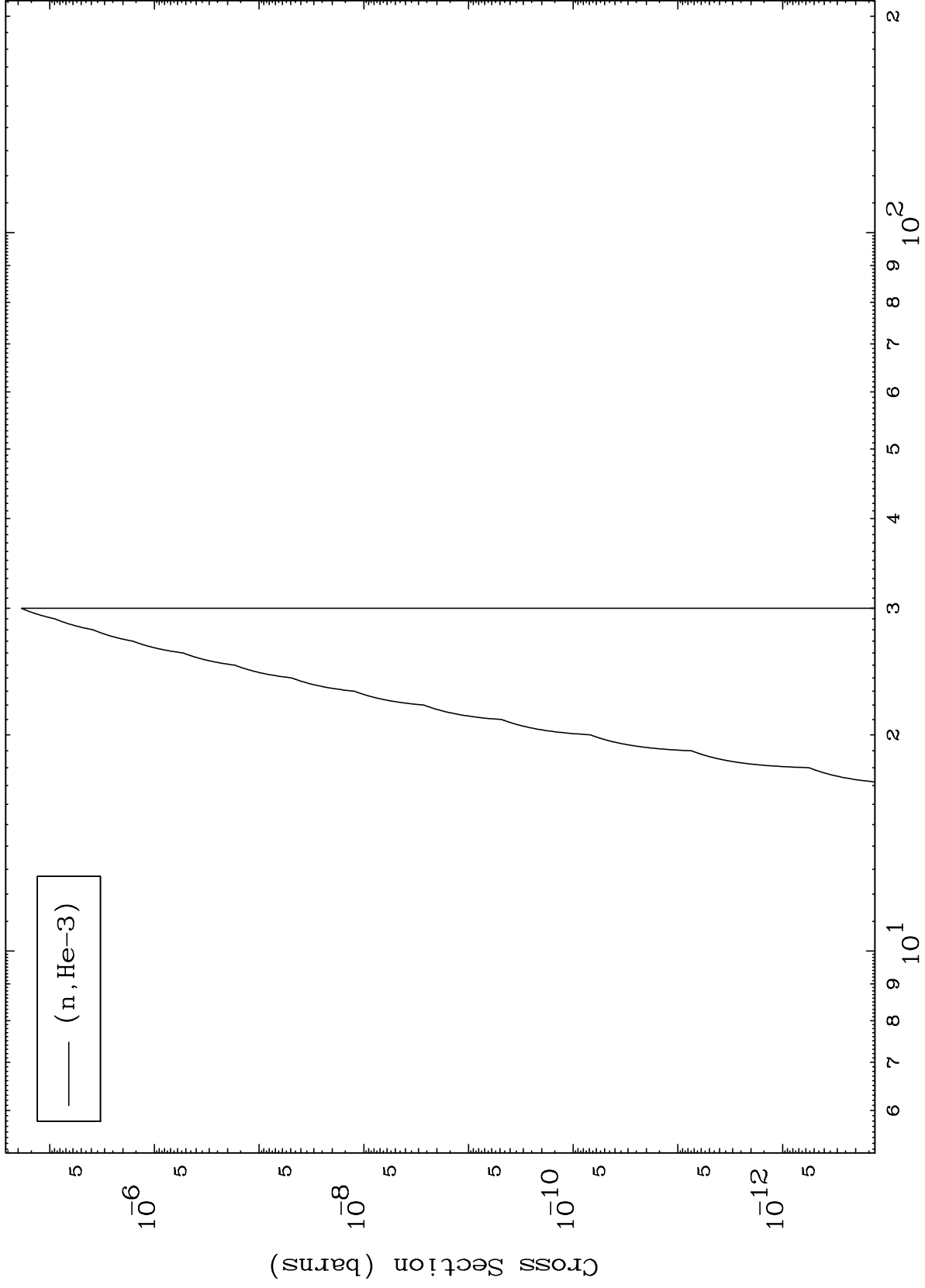
Incident Energy (MeV)

92-U -237

MAT 9234

(n,He3) Levels  
293 Kelvin Cross Sections

92-U -237



15

Incident Energy (MeV)

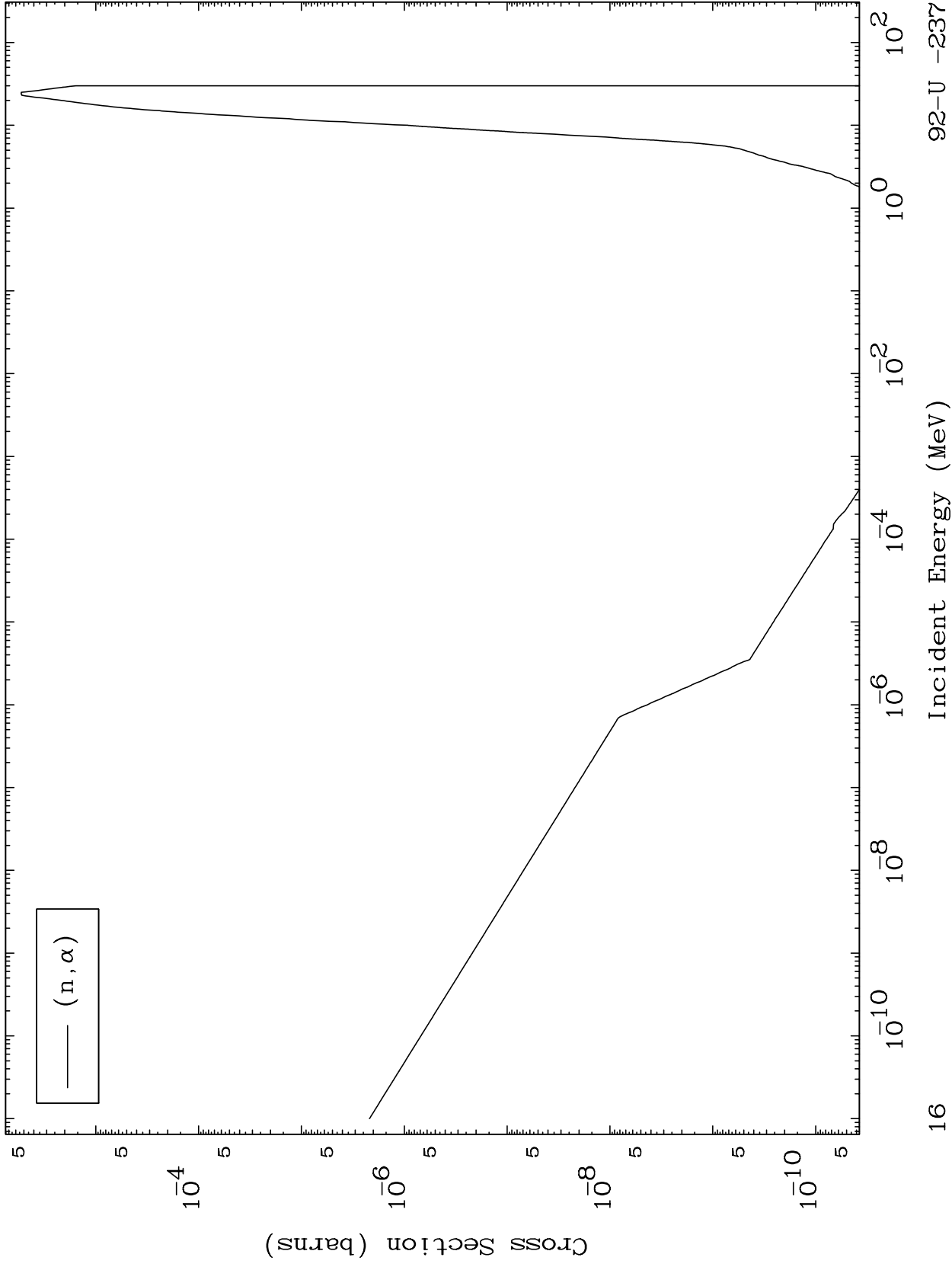
92-U -237

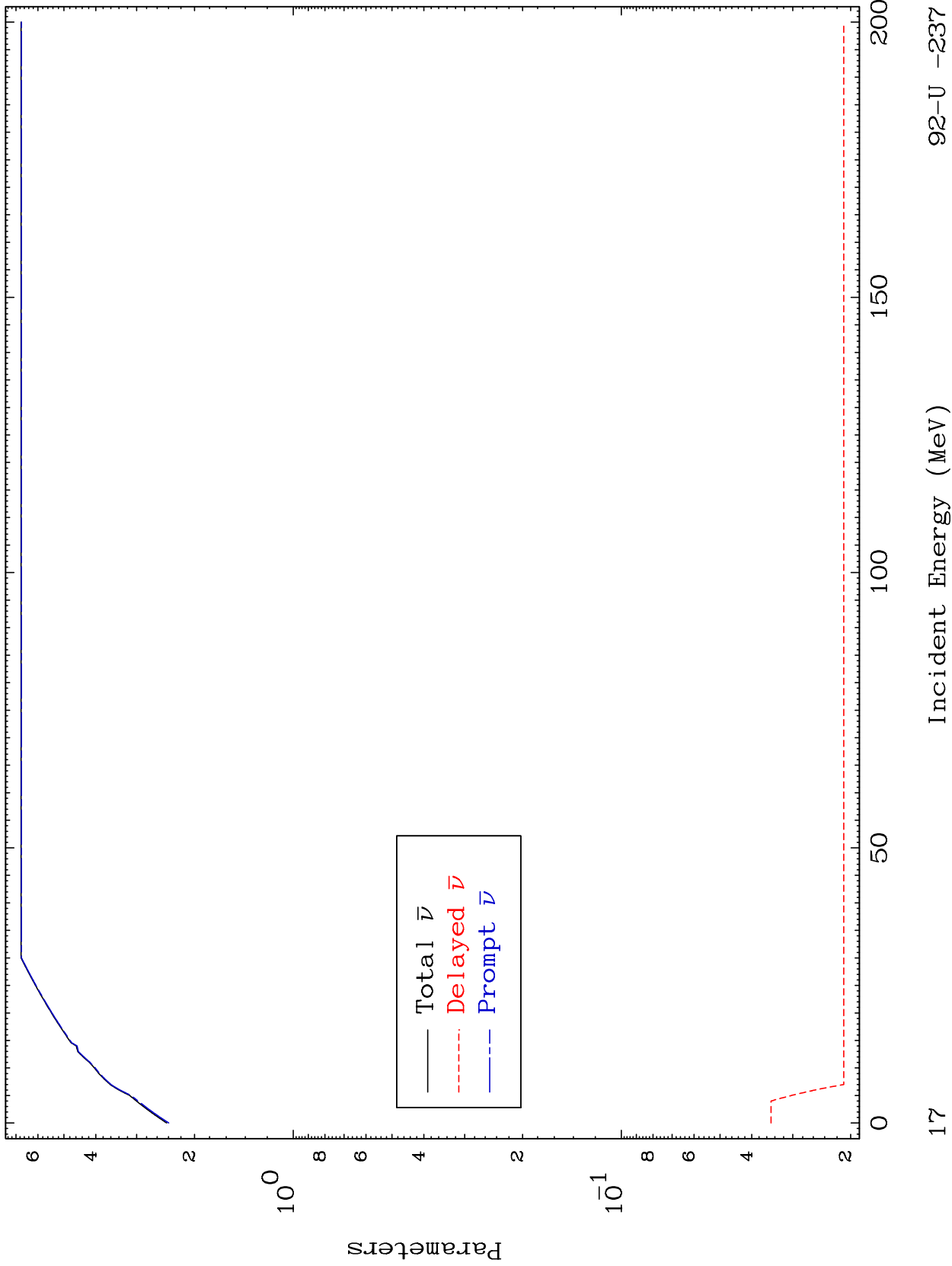


MAT 9234

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

92-U -237



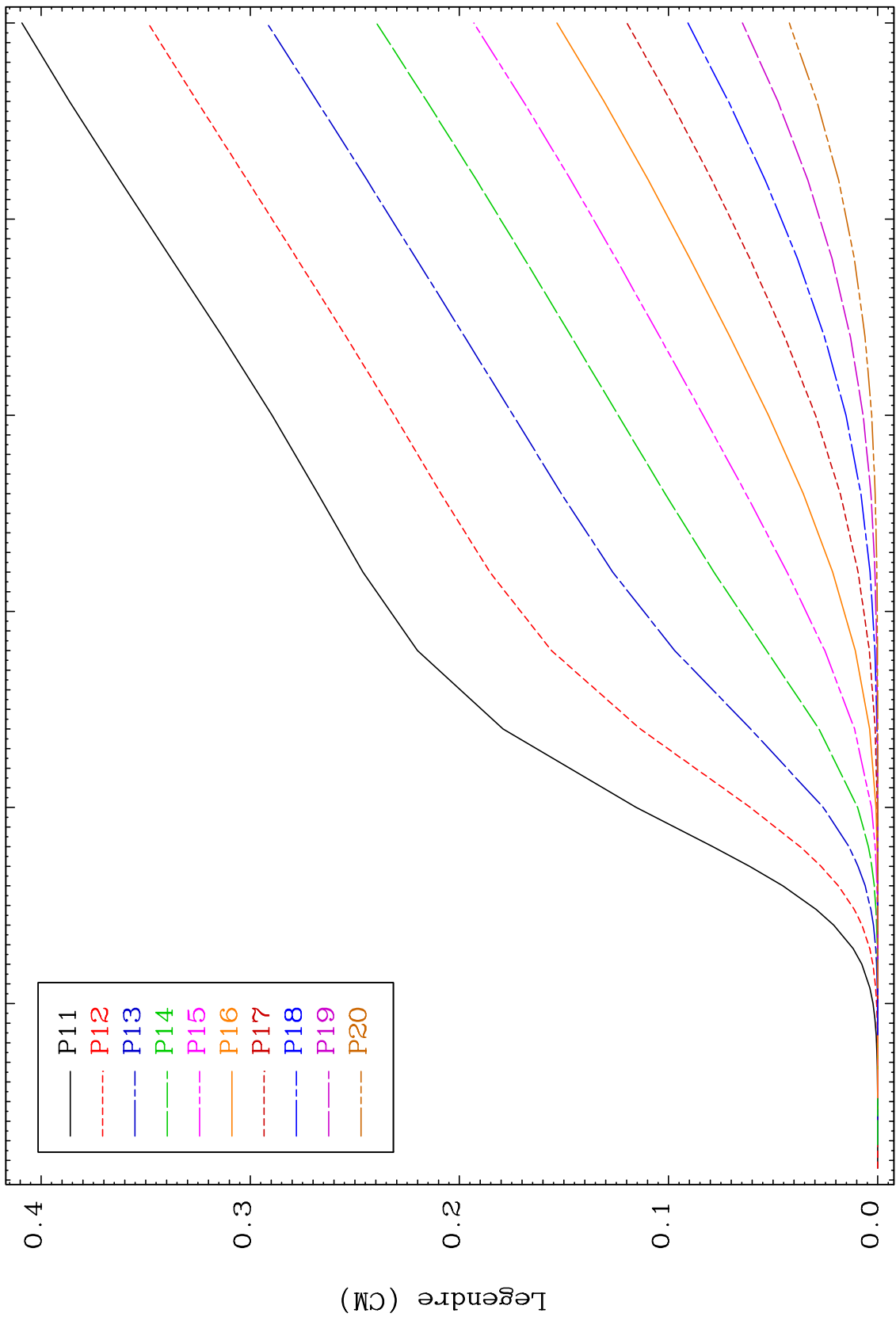




MAT 9234

Elastic Legendre Coefficients

92-U -237



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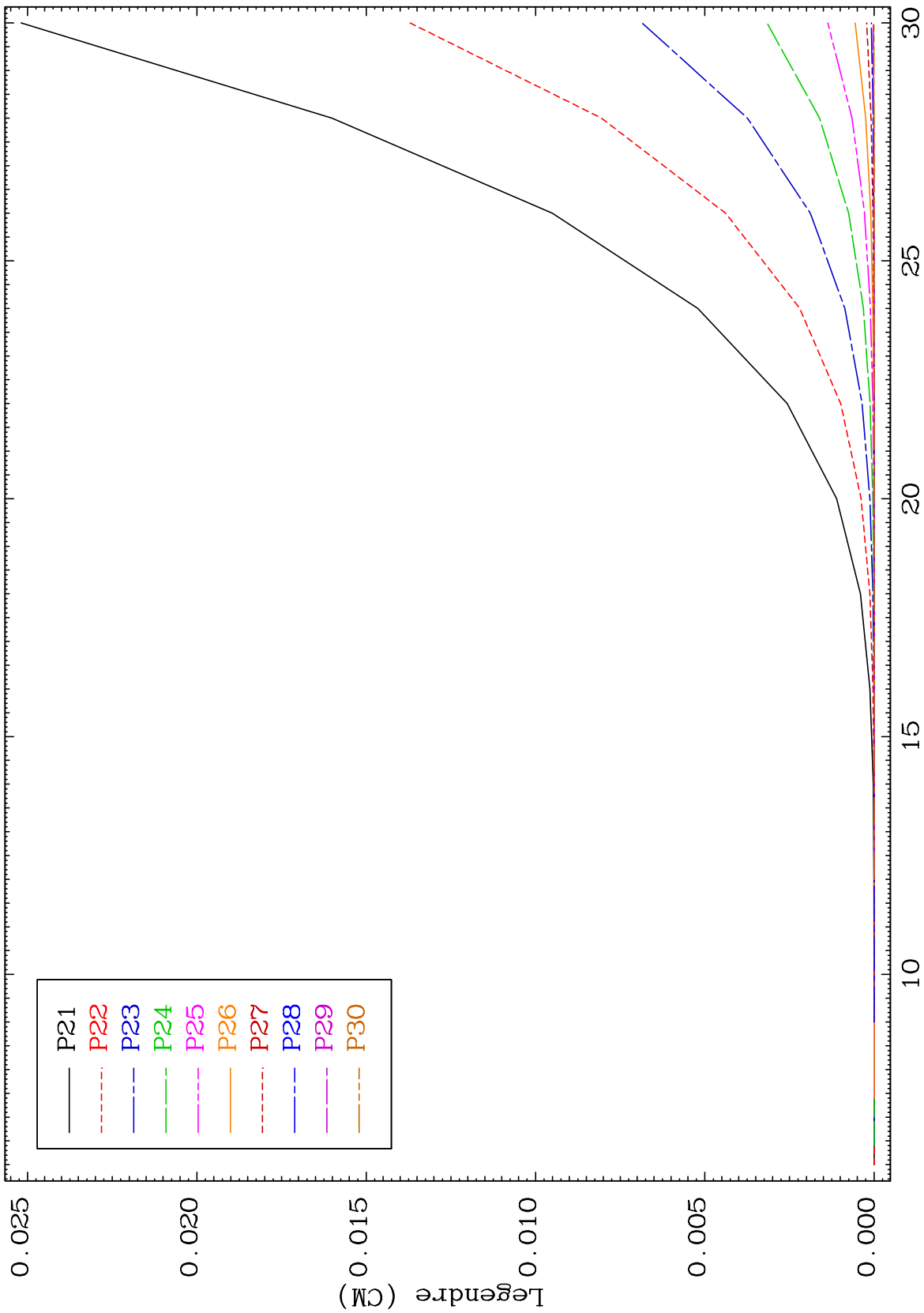
Incident Energy (MeV)

92-U -237

MAT 9234

Elastic Legendre Coefficients

92-U -237



20

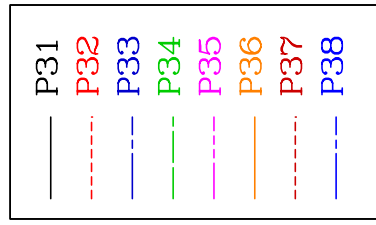
Incident Energy (MeV)

92-U -237

MAT 9234

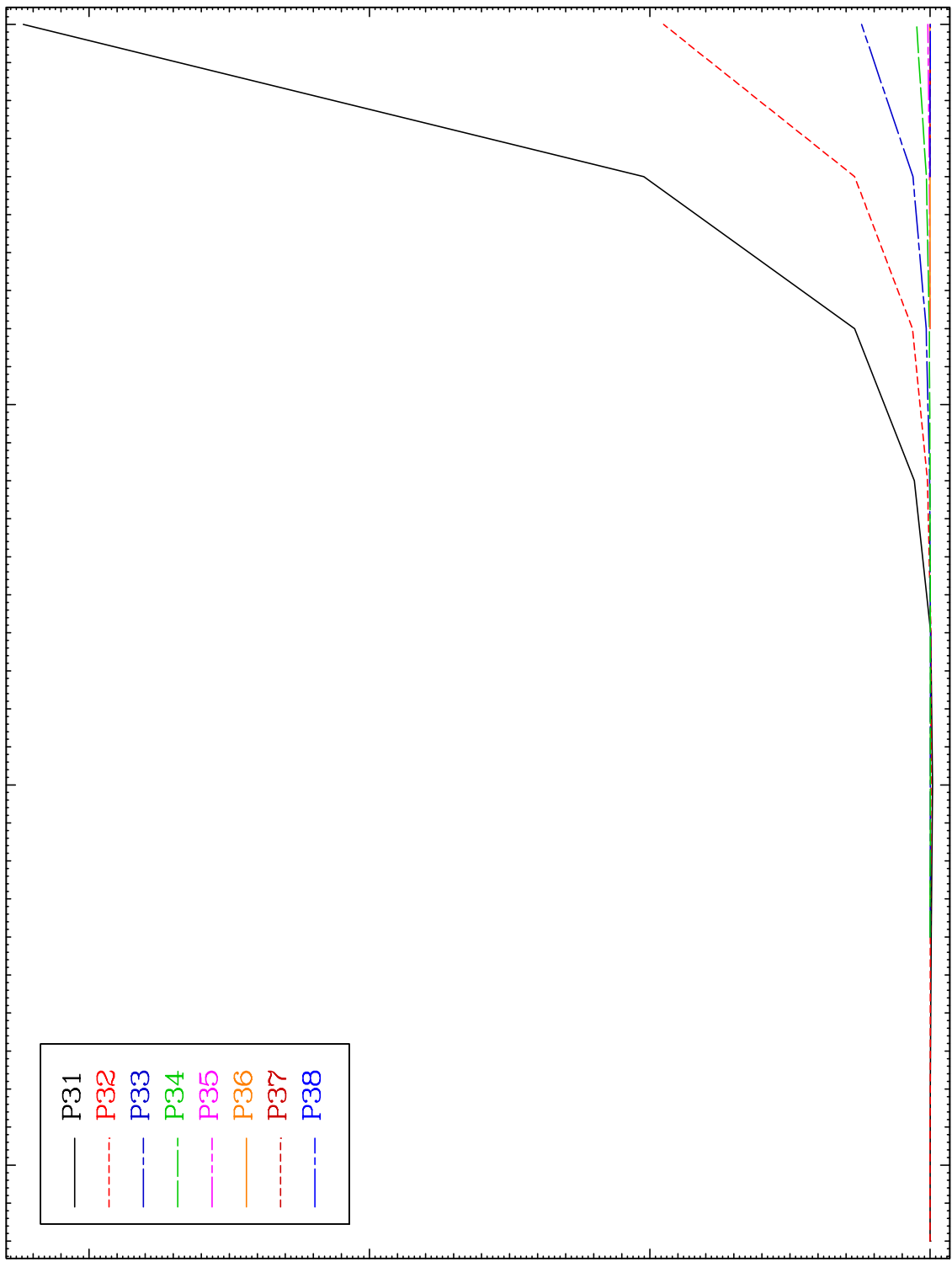
Elastic Legendre Coefficients

92-U -237



$\times 10^{-6}$

Legendre (CM)



30

25

20

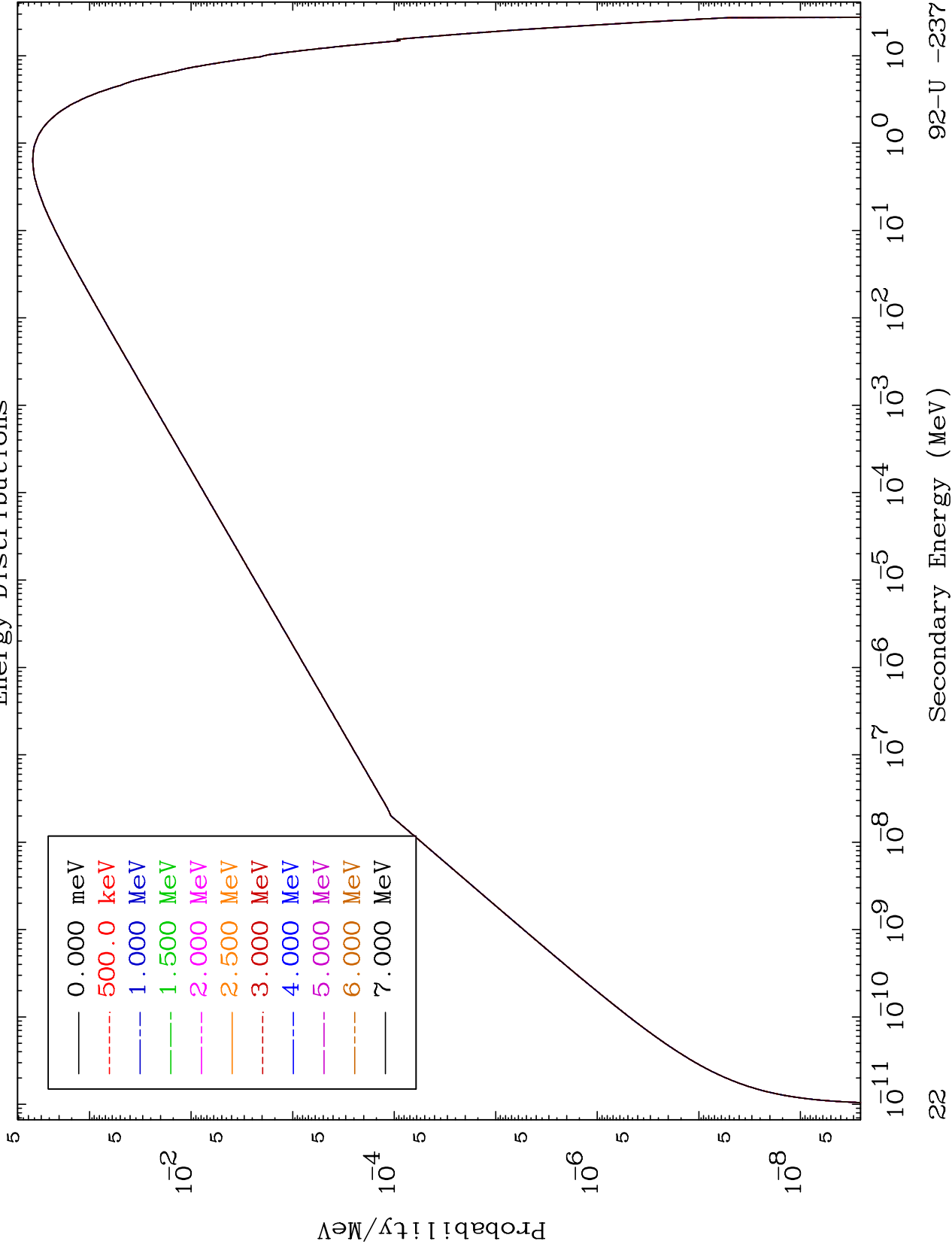
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21

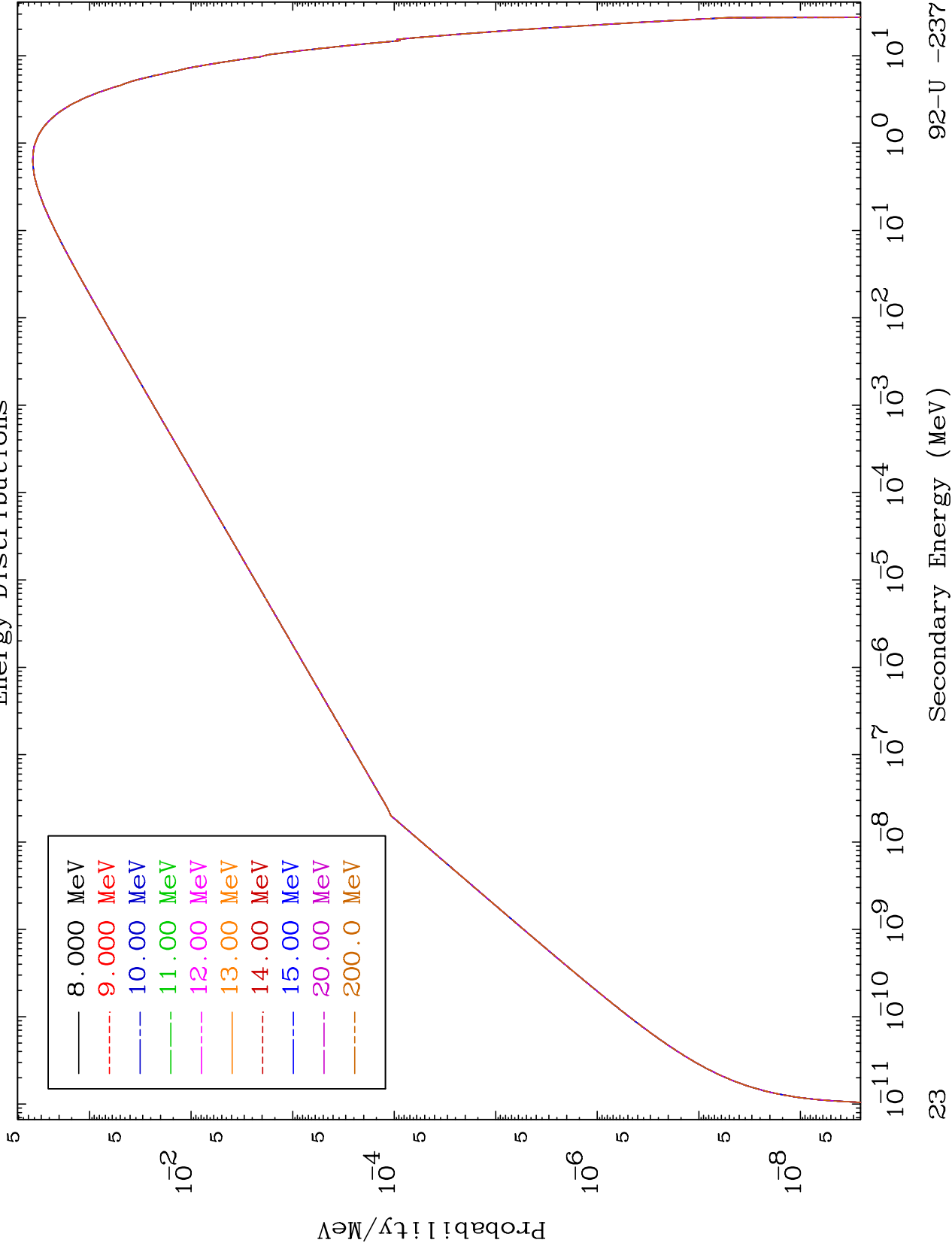
Incident Energy (MeV)

92-U -237

Fission Energy Distributions



Fission Energy Distributions



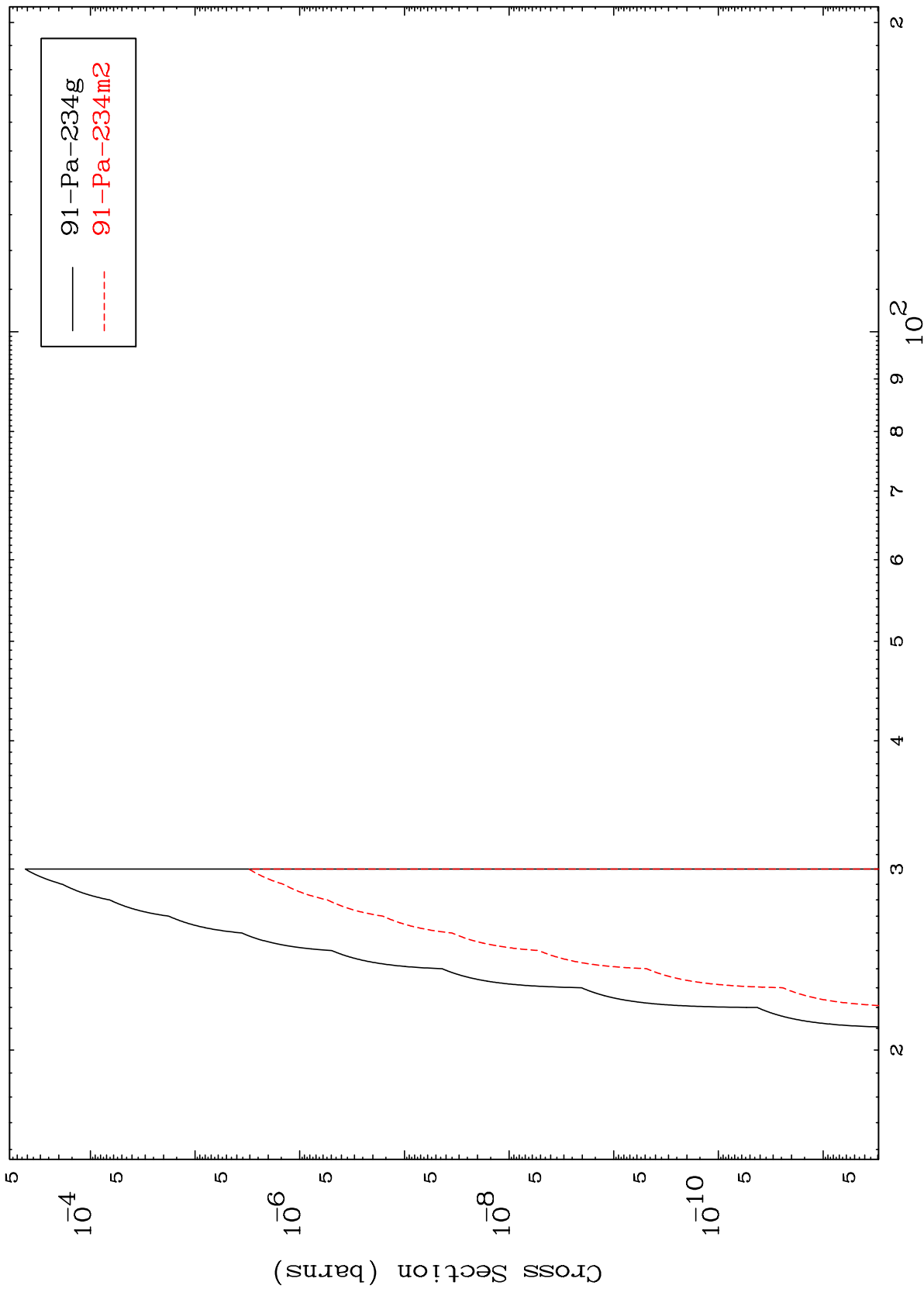


MAT 92334

(n,2n) d

92-U -237

Radionuclide Production Cross Section



24

Incident Energy (MeV)

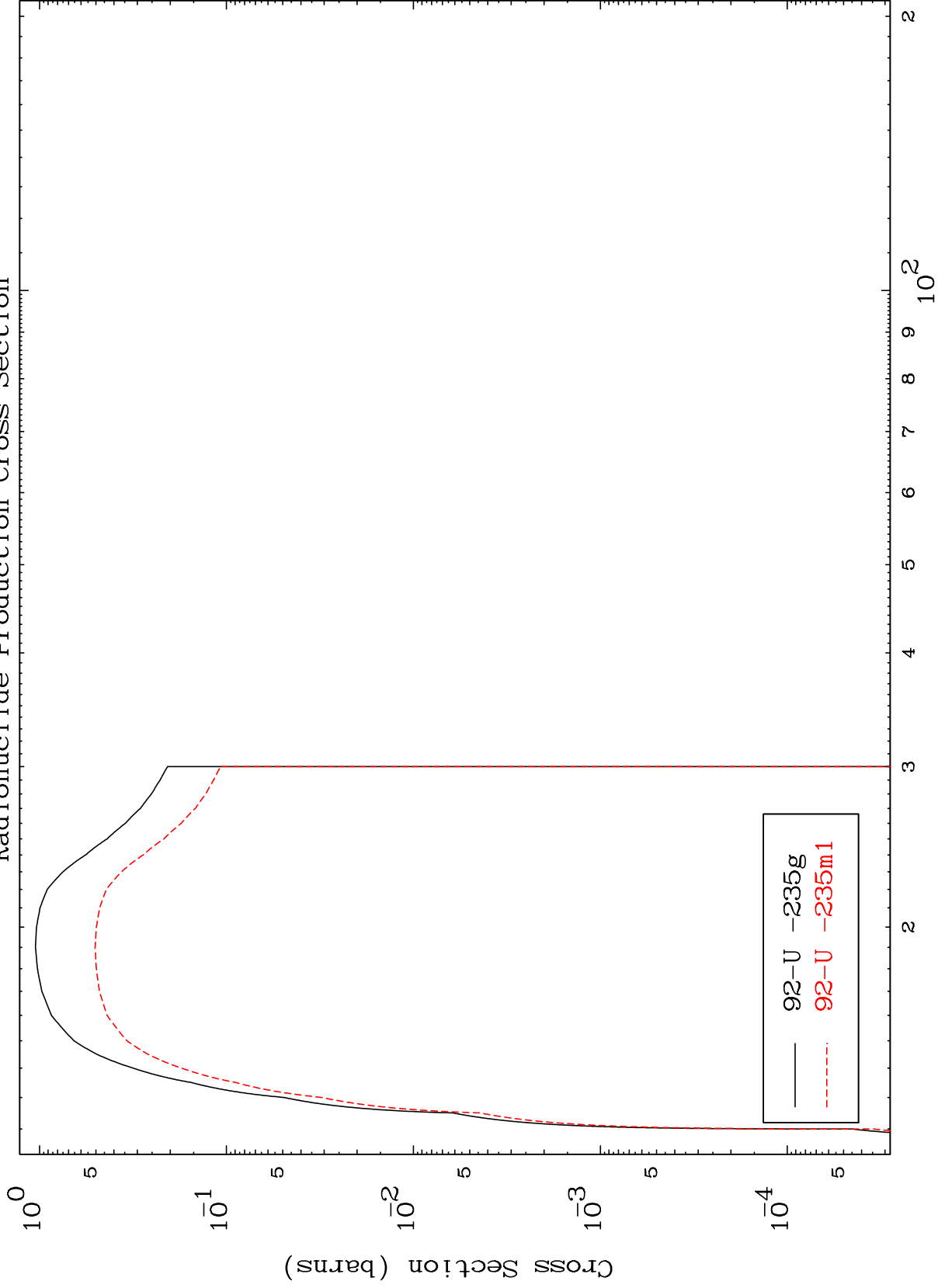
92-U -237

MAT 9234

(n,3n)

92-U -237

Radionuclide Production Cross Section



Incident Energy (MeV)

92-U -237

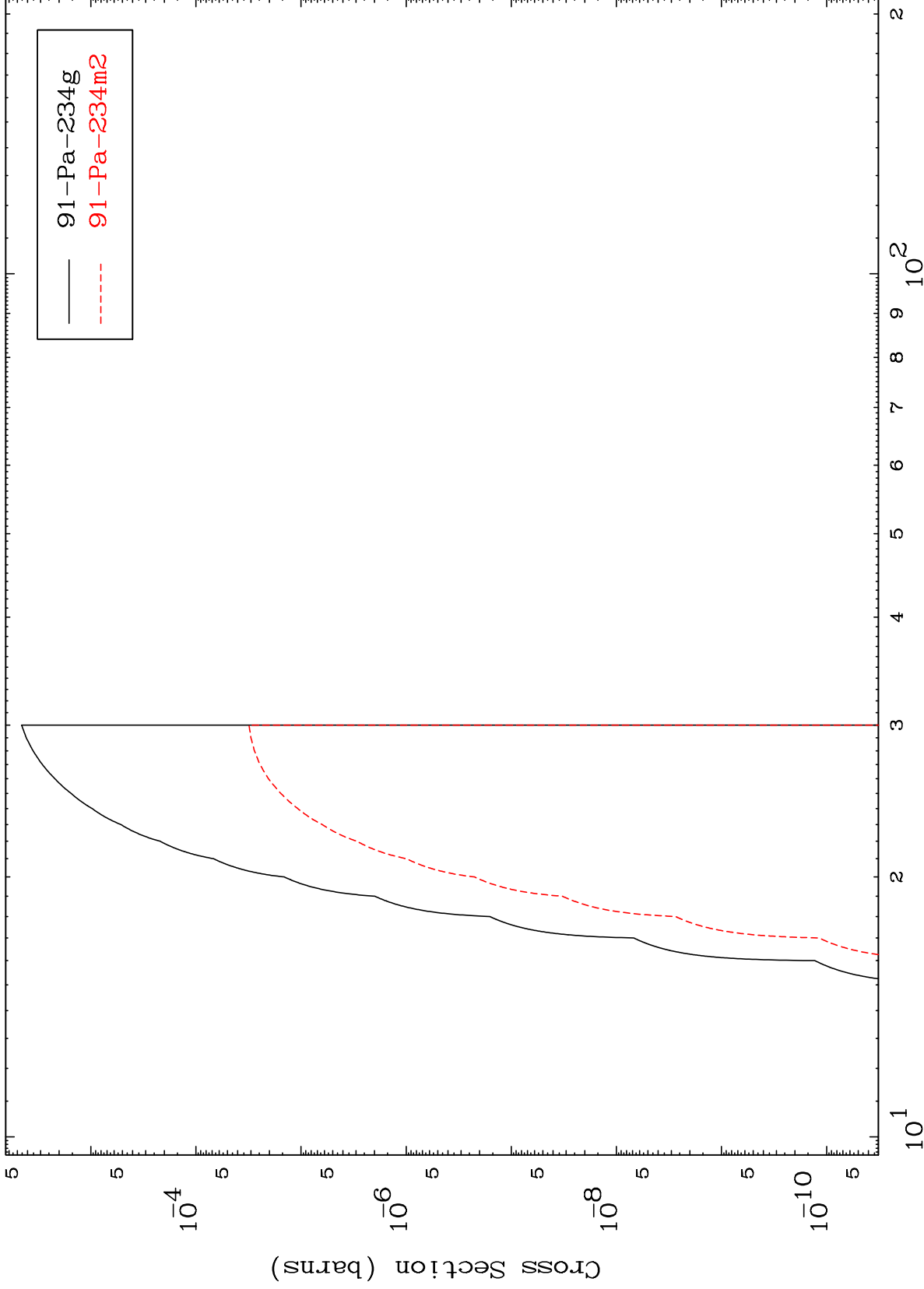
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MAT 9234

(n,n') t

92-U -237

Radionuclide Production Cross Section



26

Incident Energy (MeV)

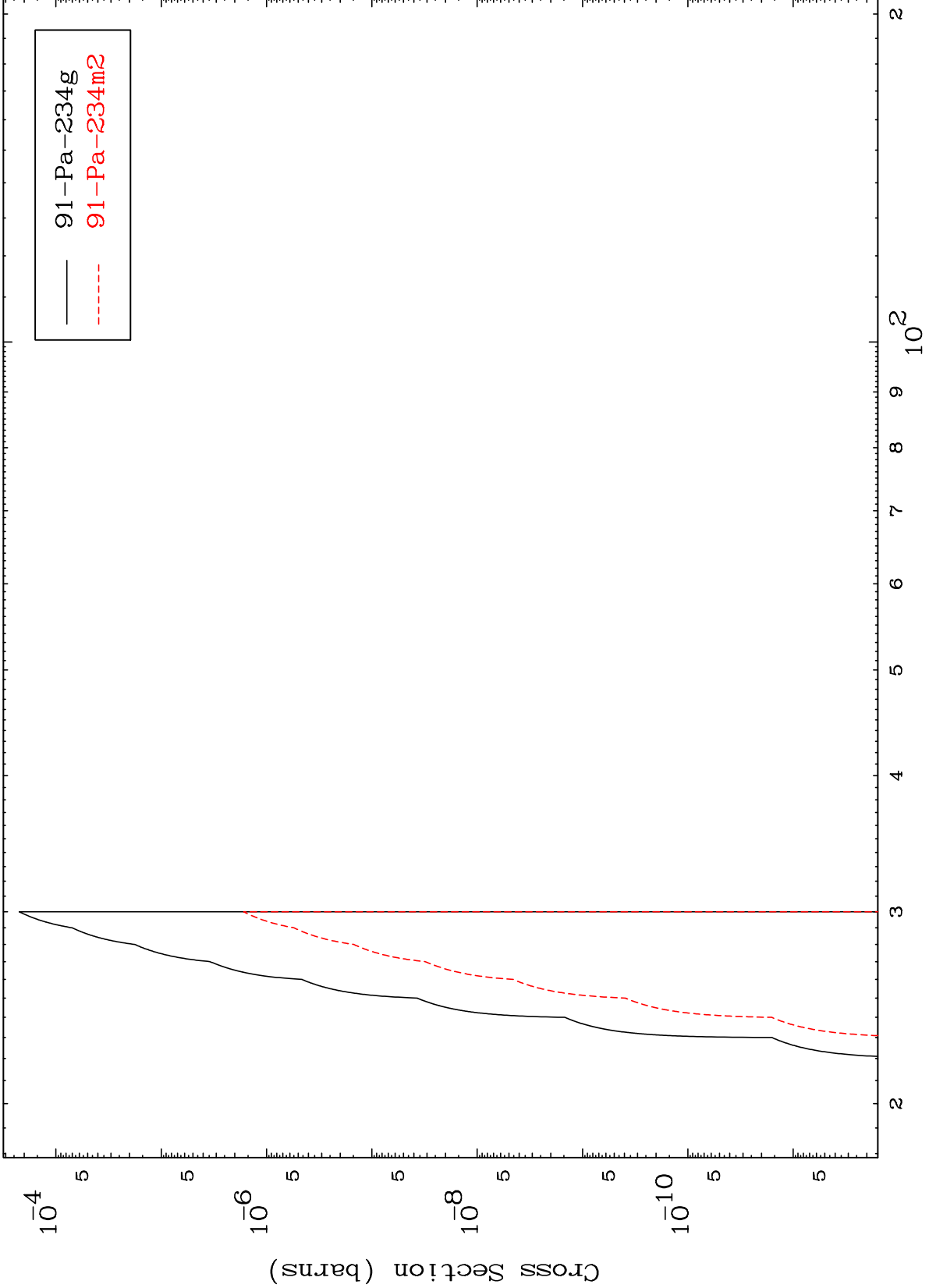
92-U -237

MAT 92334

(n,3n) p

92-U -237

Radionuclide Production Cross Section



91-Pa-234g  
91-Pa-234m2

27

Incident Energy (MeV)

92-U -237