

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

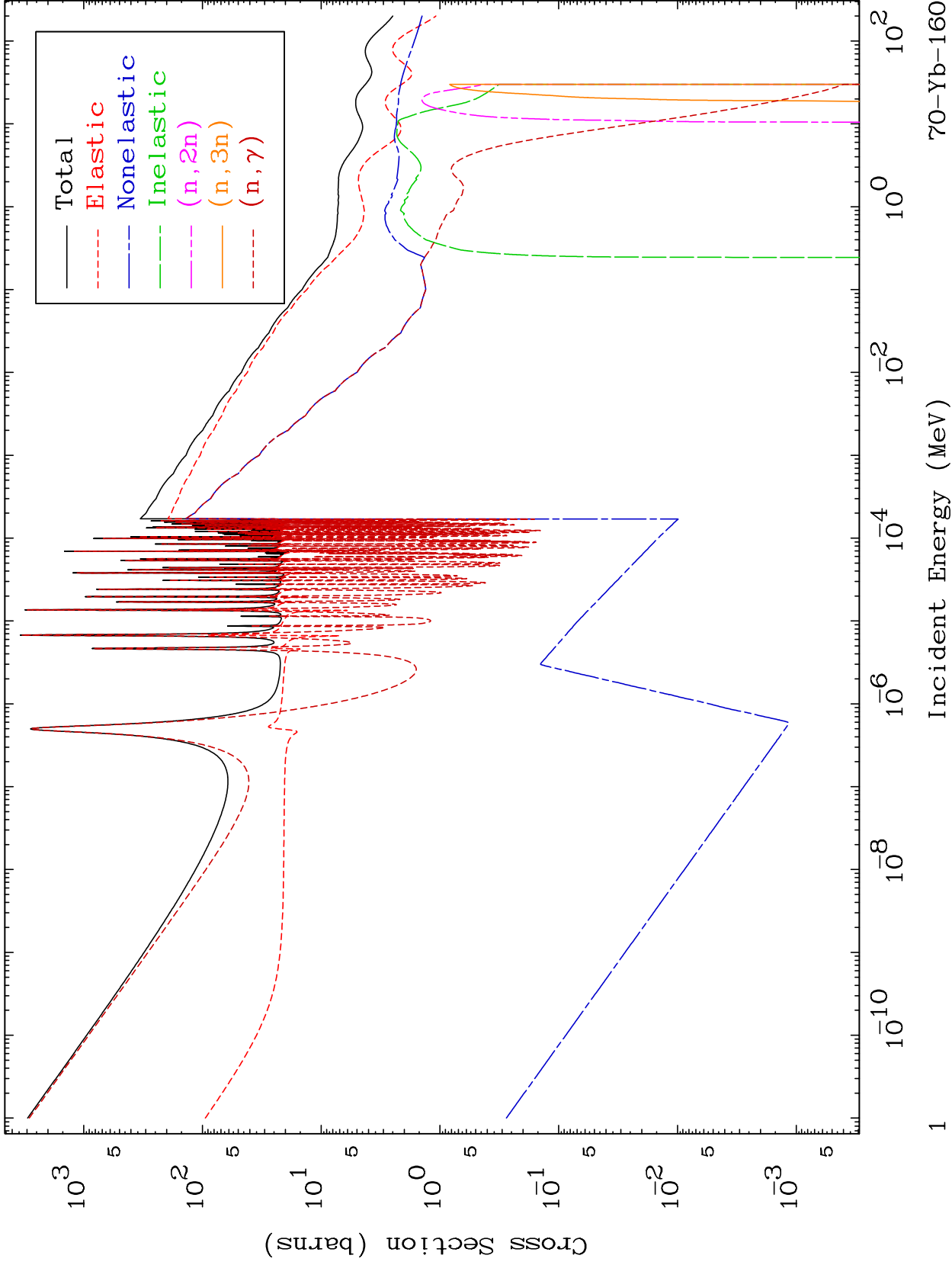
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7001

Neutron Major
293 Kelvin Cross Sections

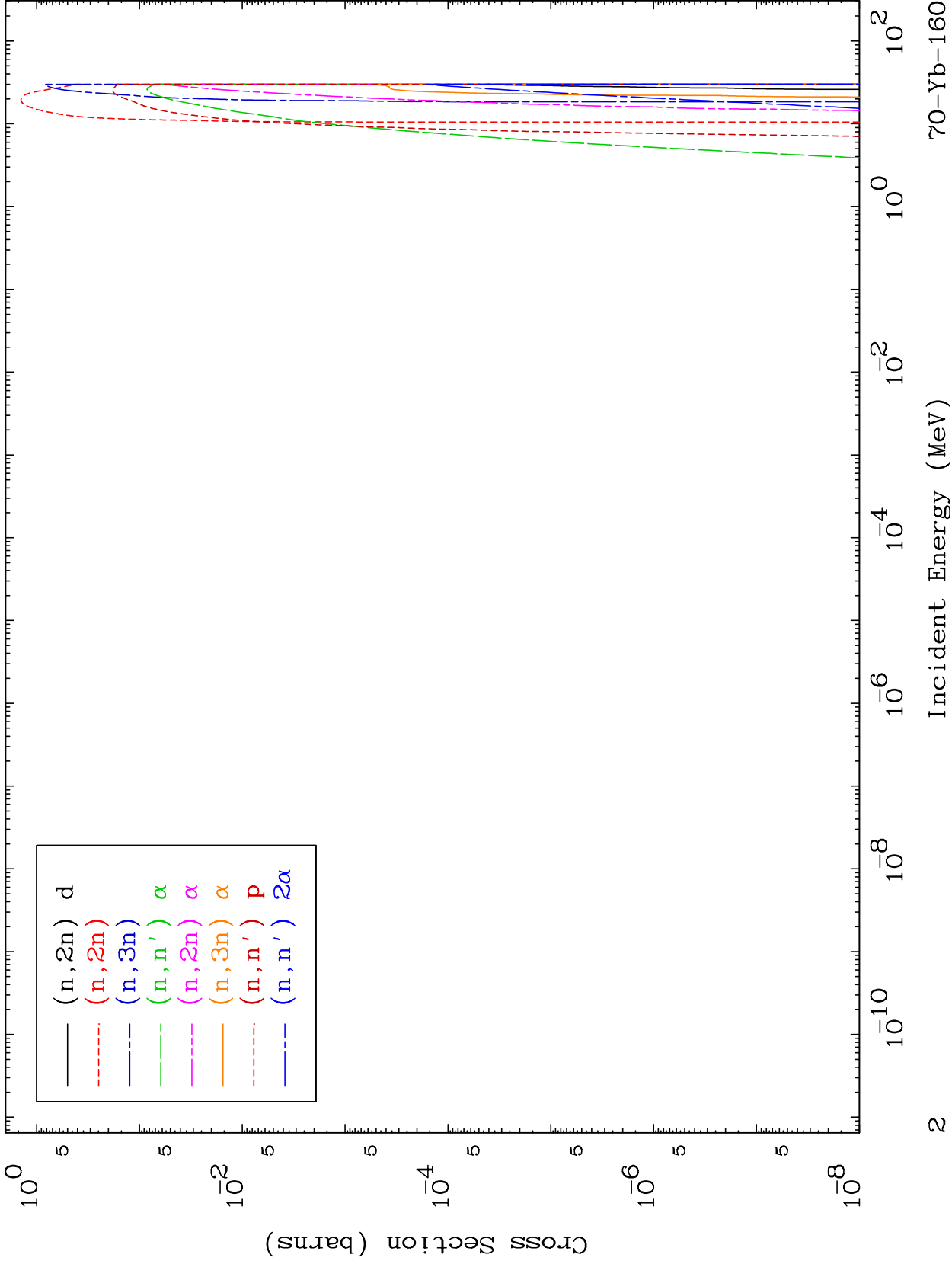
70-Yb-160



MAT 7001

Neutron Absorption
293 Kelvin Cross Sections

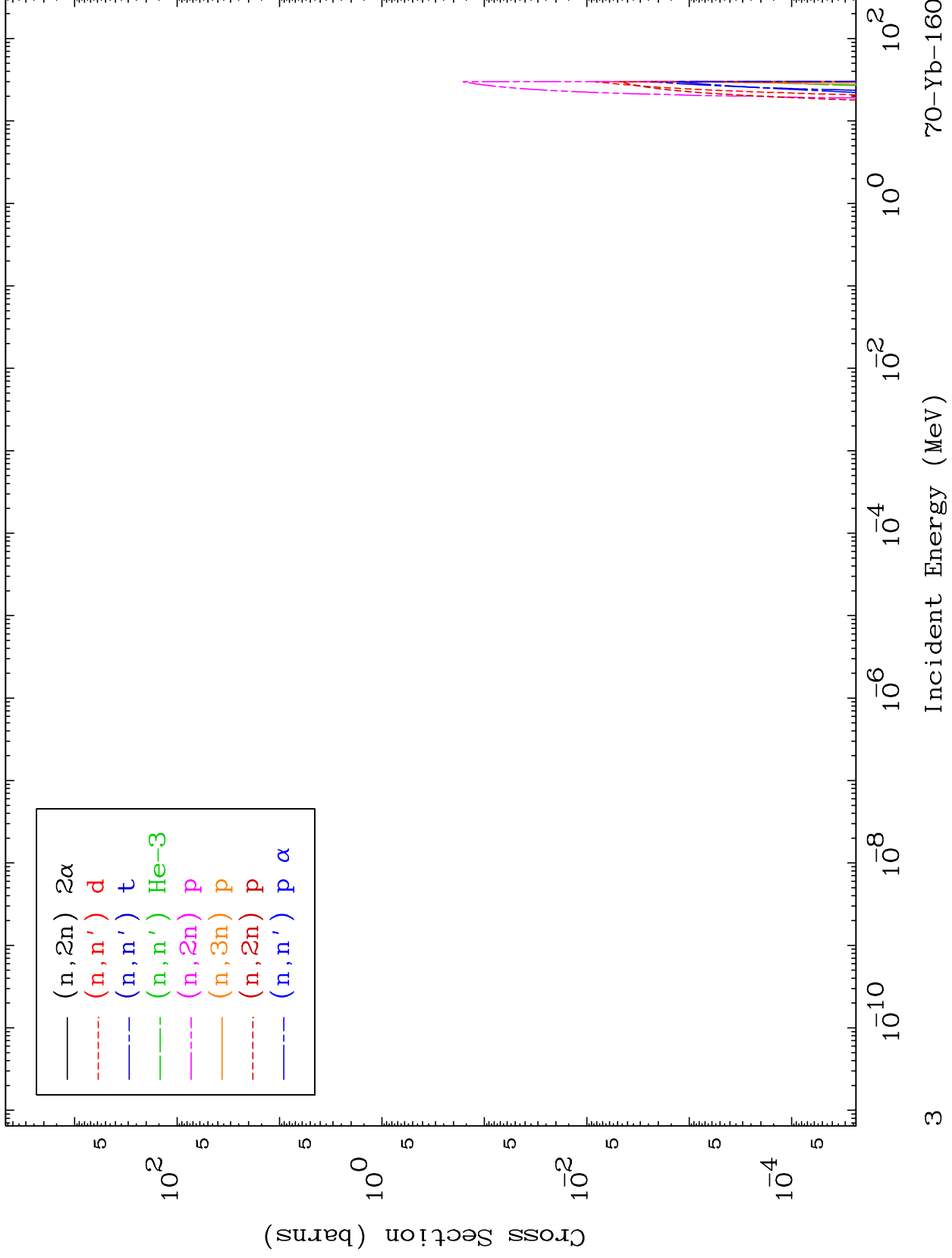
70-Yb-160

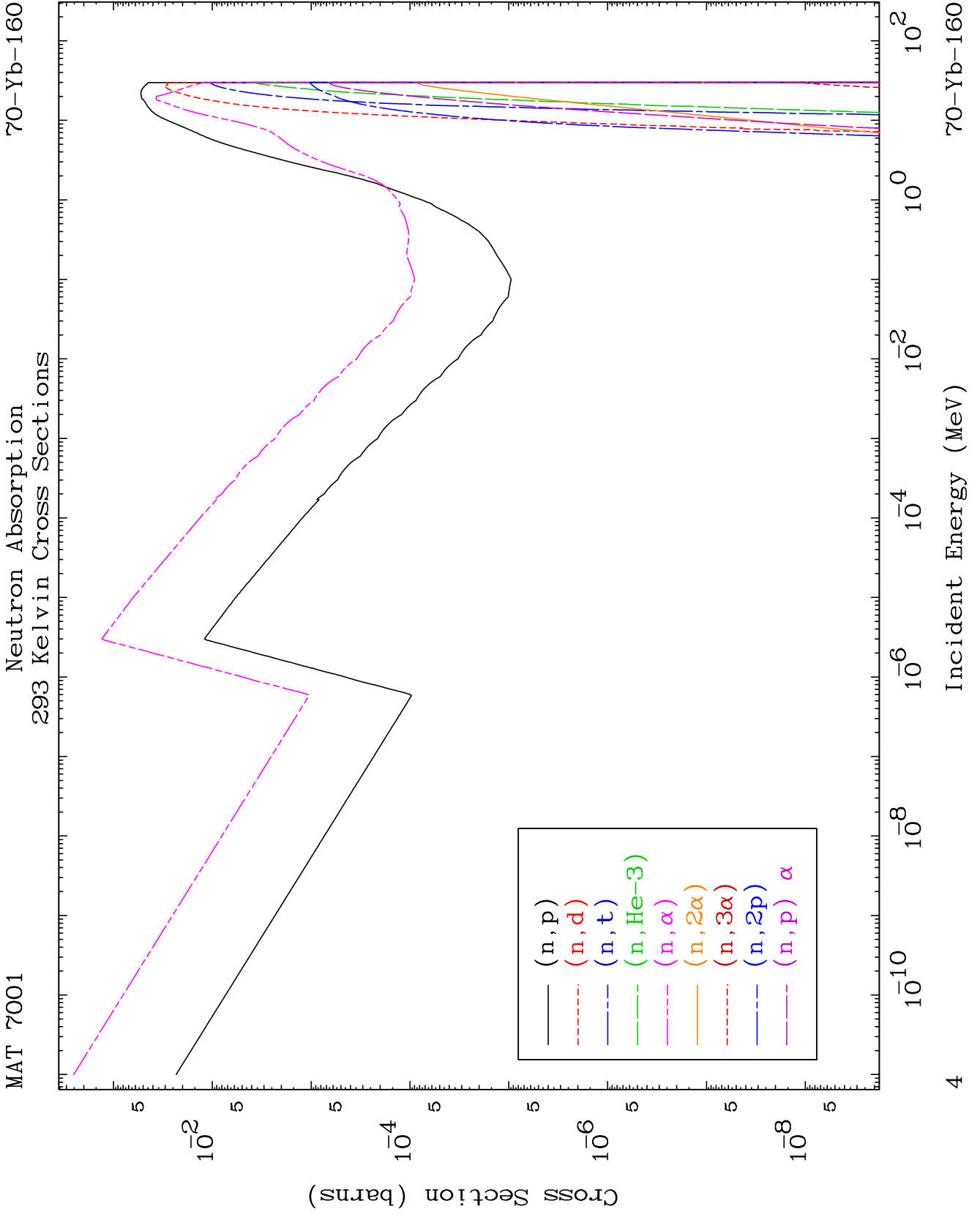


MAT 7001

Neutron Absorption
293 Kelvin Cross Sections

70-Yb-160

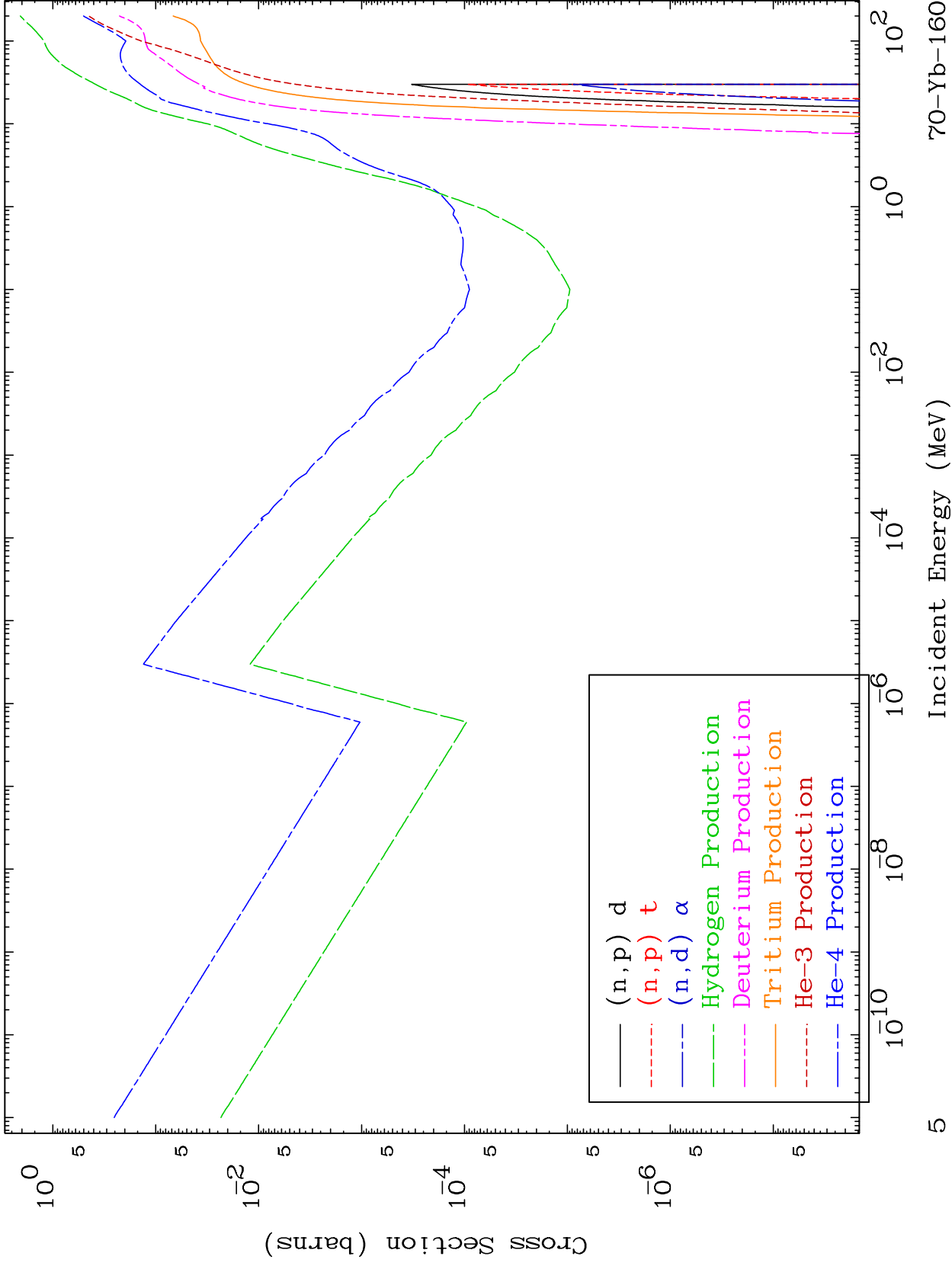




MAT 7001

Neutron Absorption
293 Kelvin Cross Sections

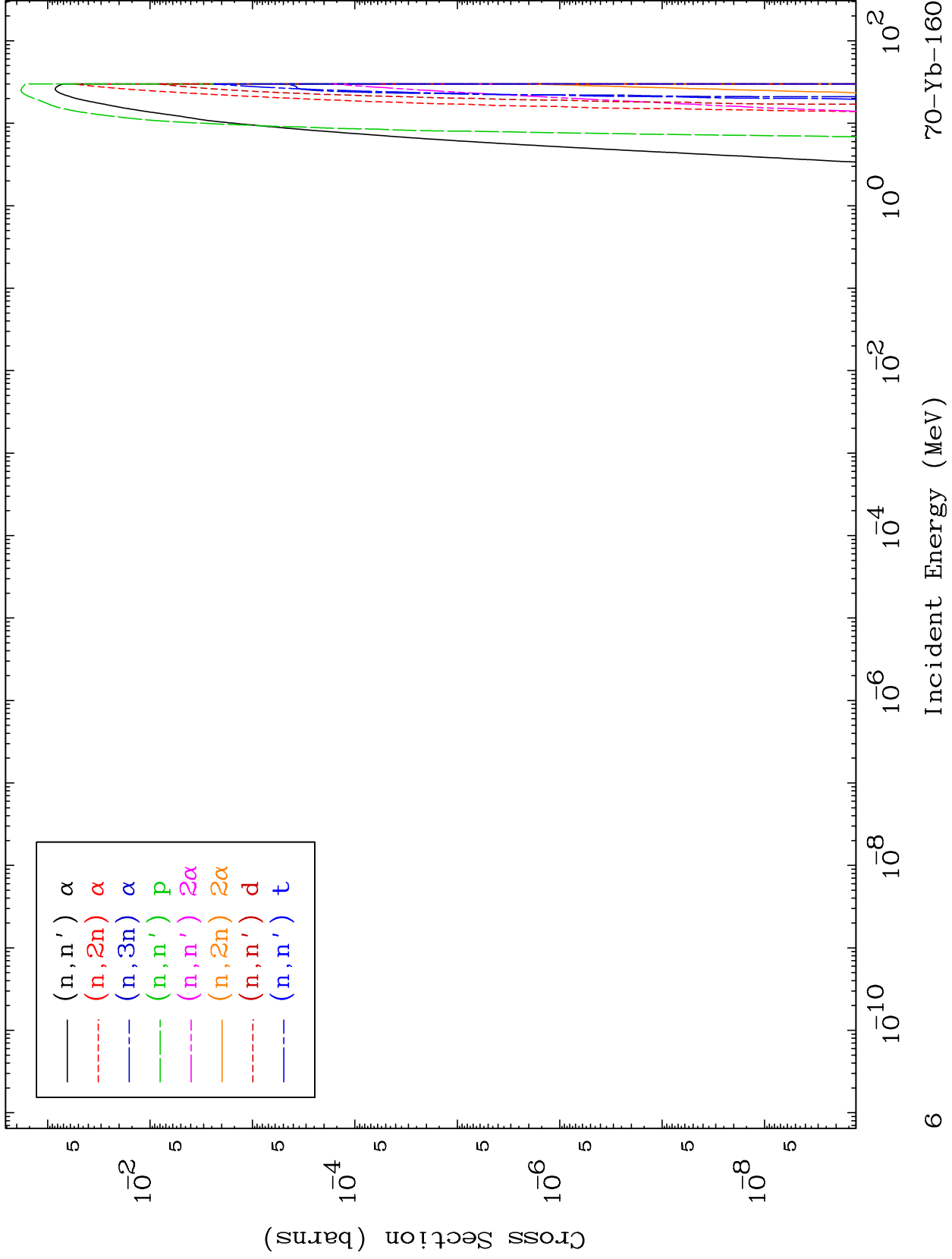
70-Yb-160

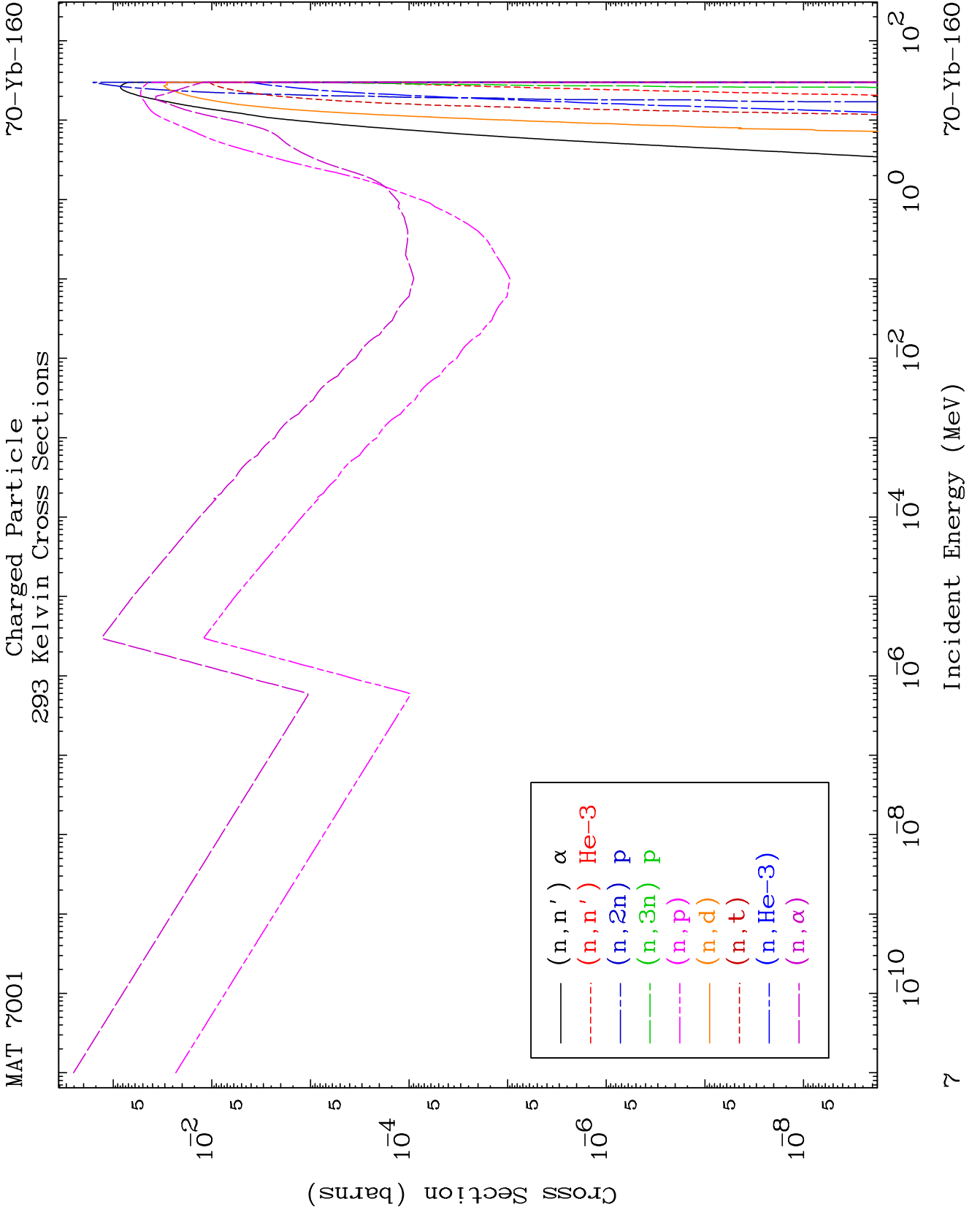


MAT 7001

Charged Particle
293 Kelvin Cross Sections

70-Yb-160

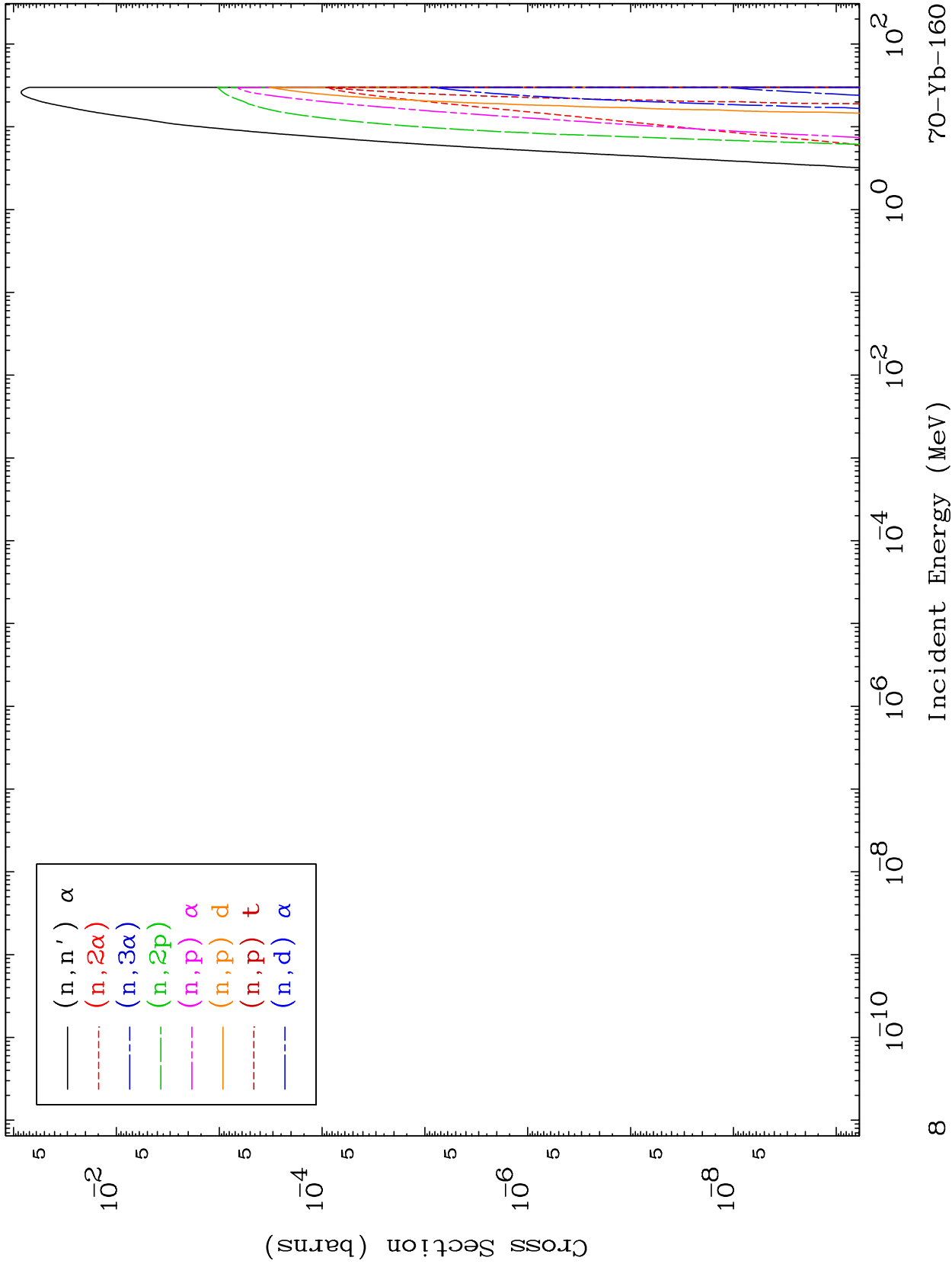




MAT 7001

Charged Particle
293 Kelvin Cross Sections

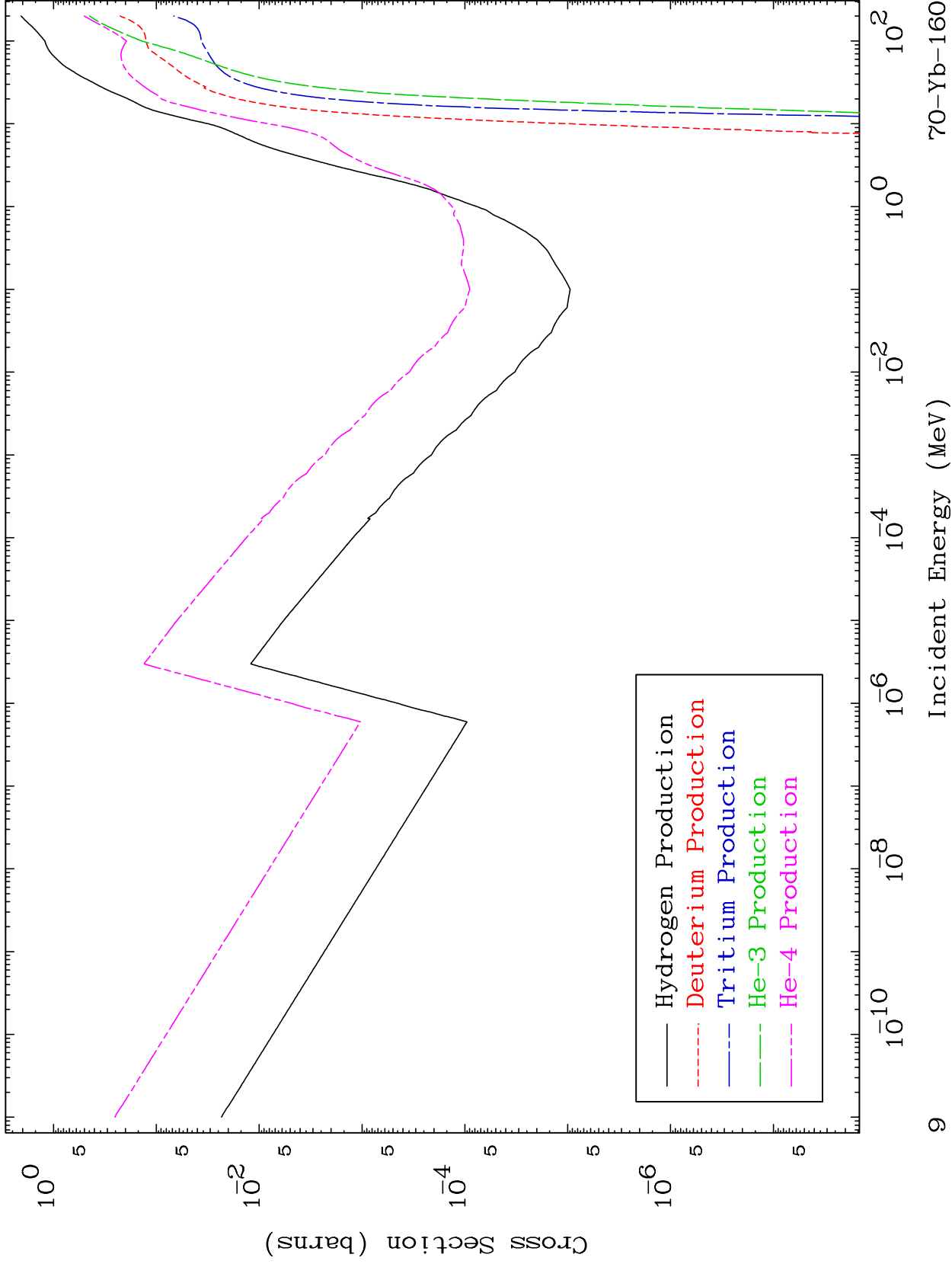
70-Yb-160



MAT 7001

Particle Production
293 Kelvin Cross Sections

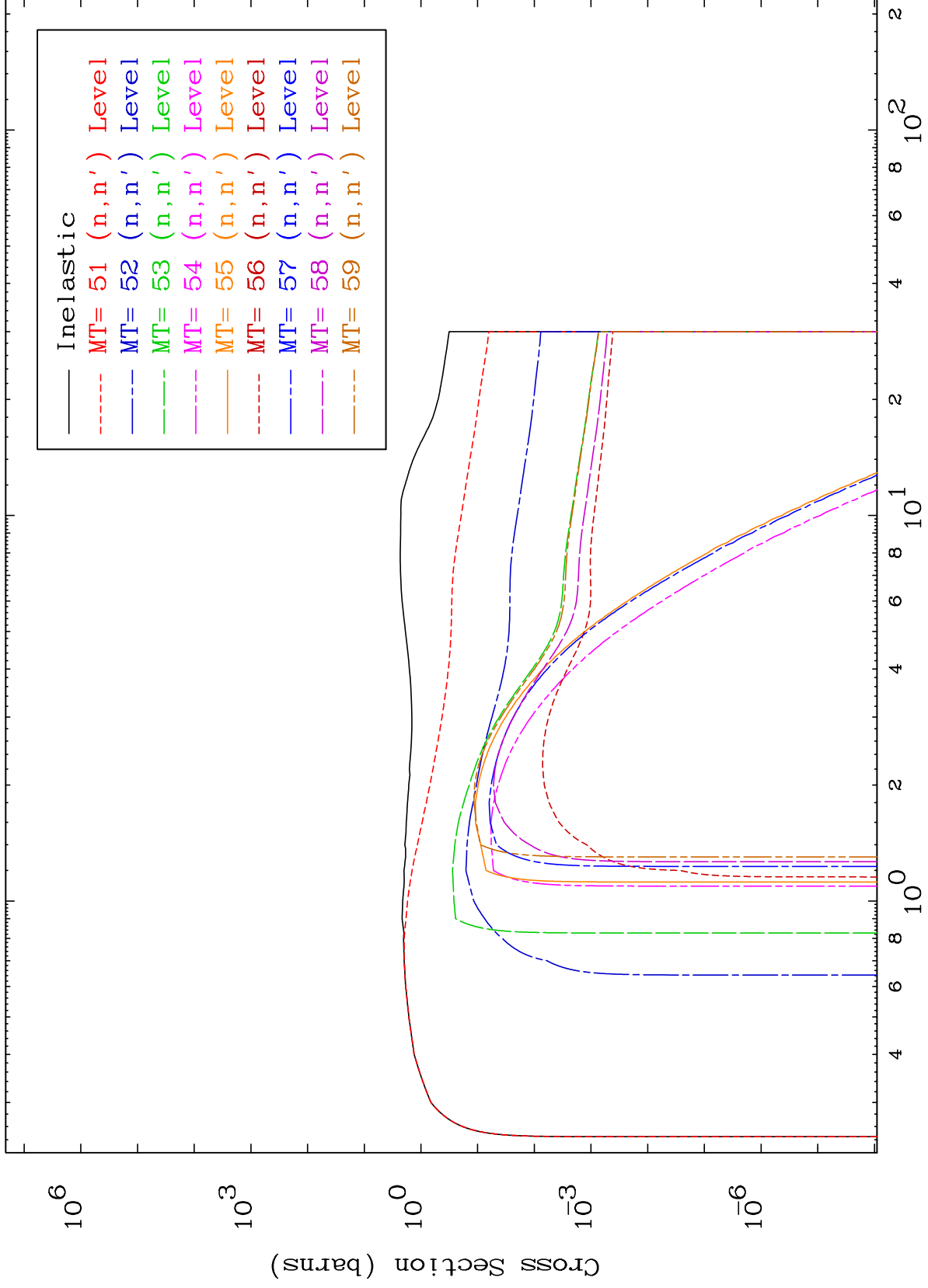
70-Yb-160



MAT 7001

(n,n') Levels
293 Kelvin Cross Sections

70-Yb-160

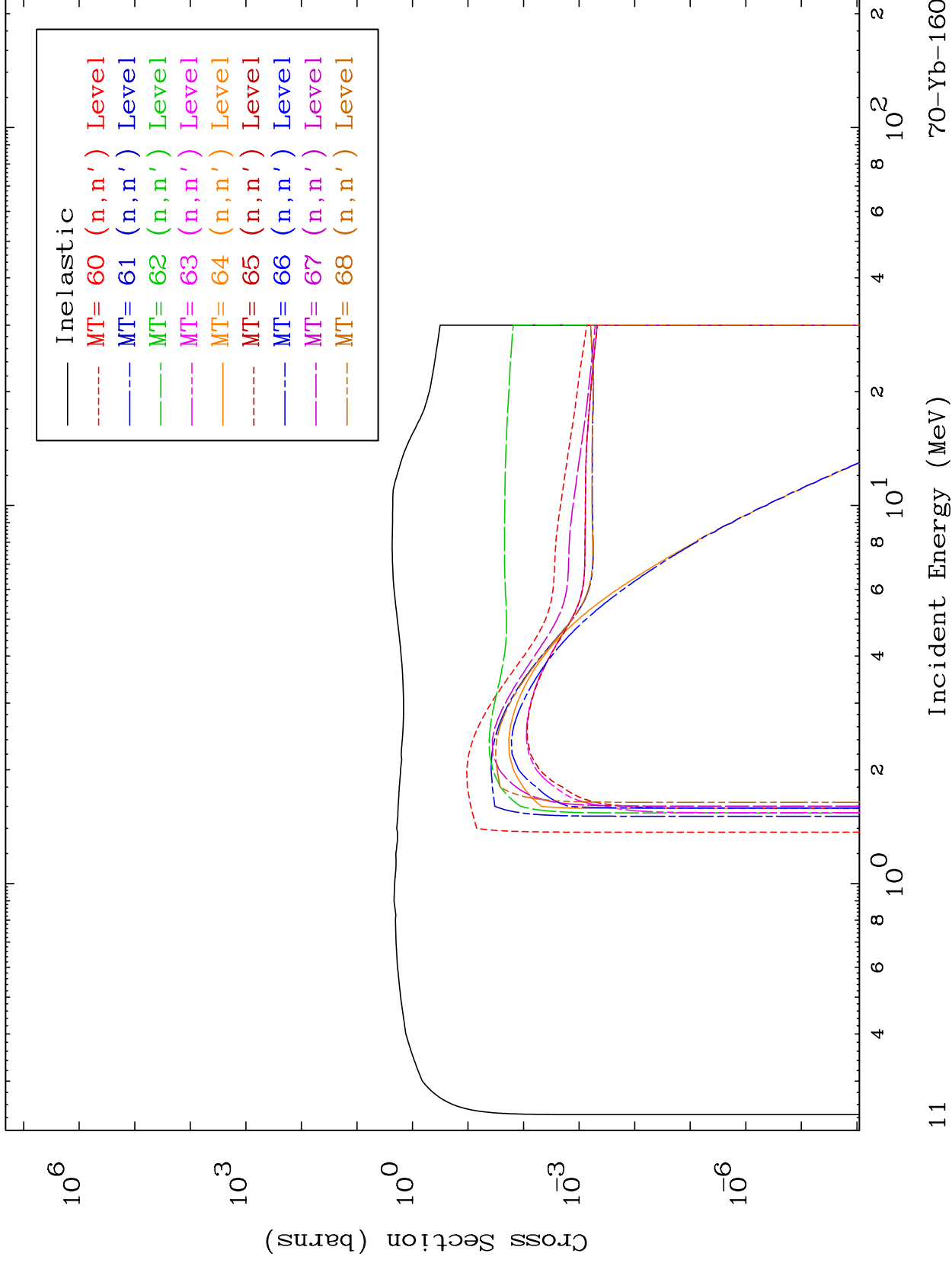


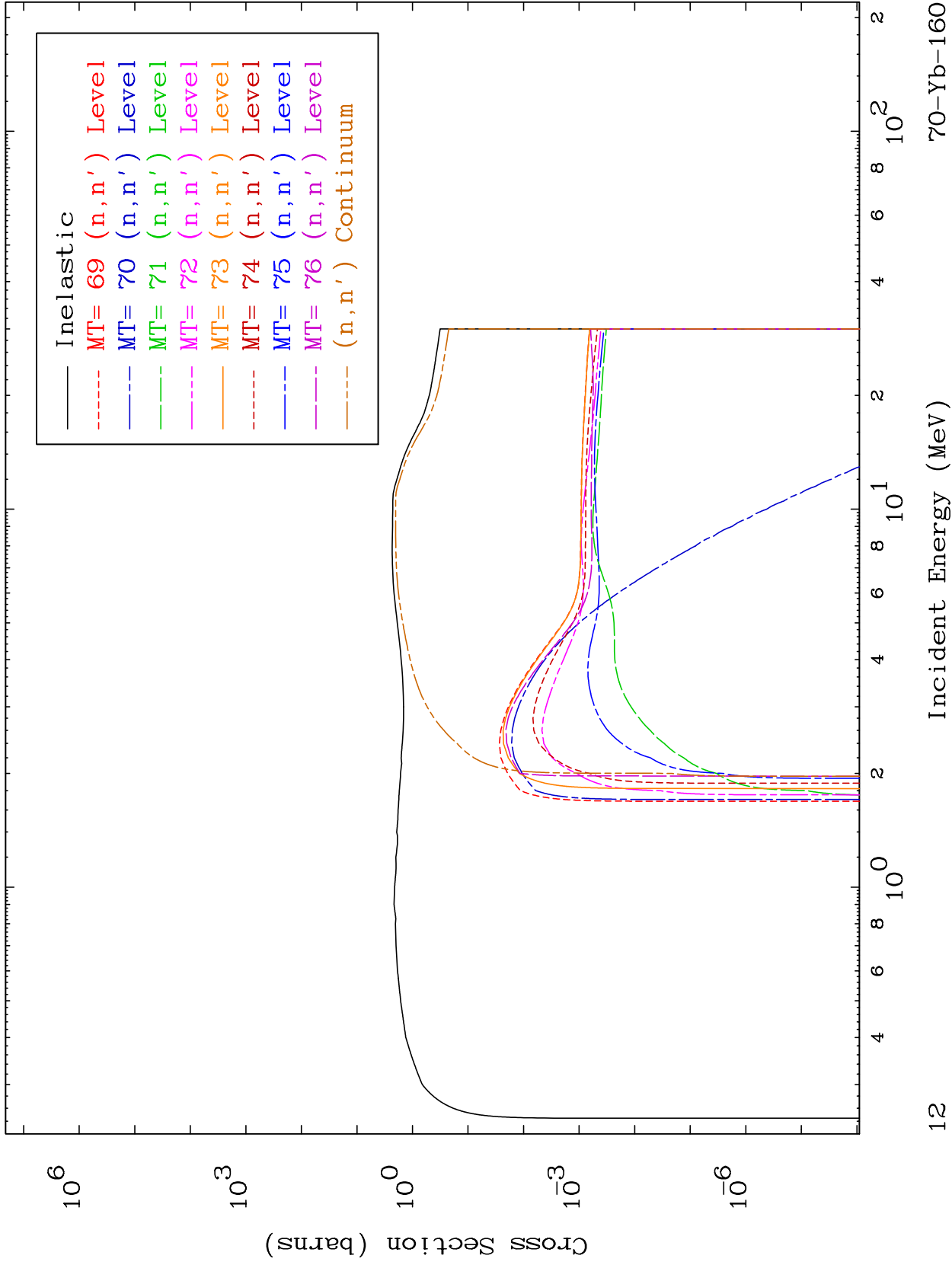
10

Incident Energy (MeV)

70-Yb-160

293 Kelvin Cross Sections

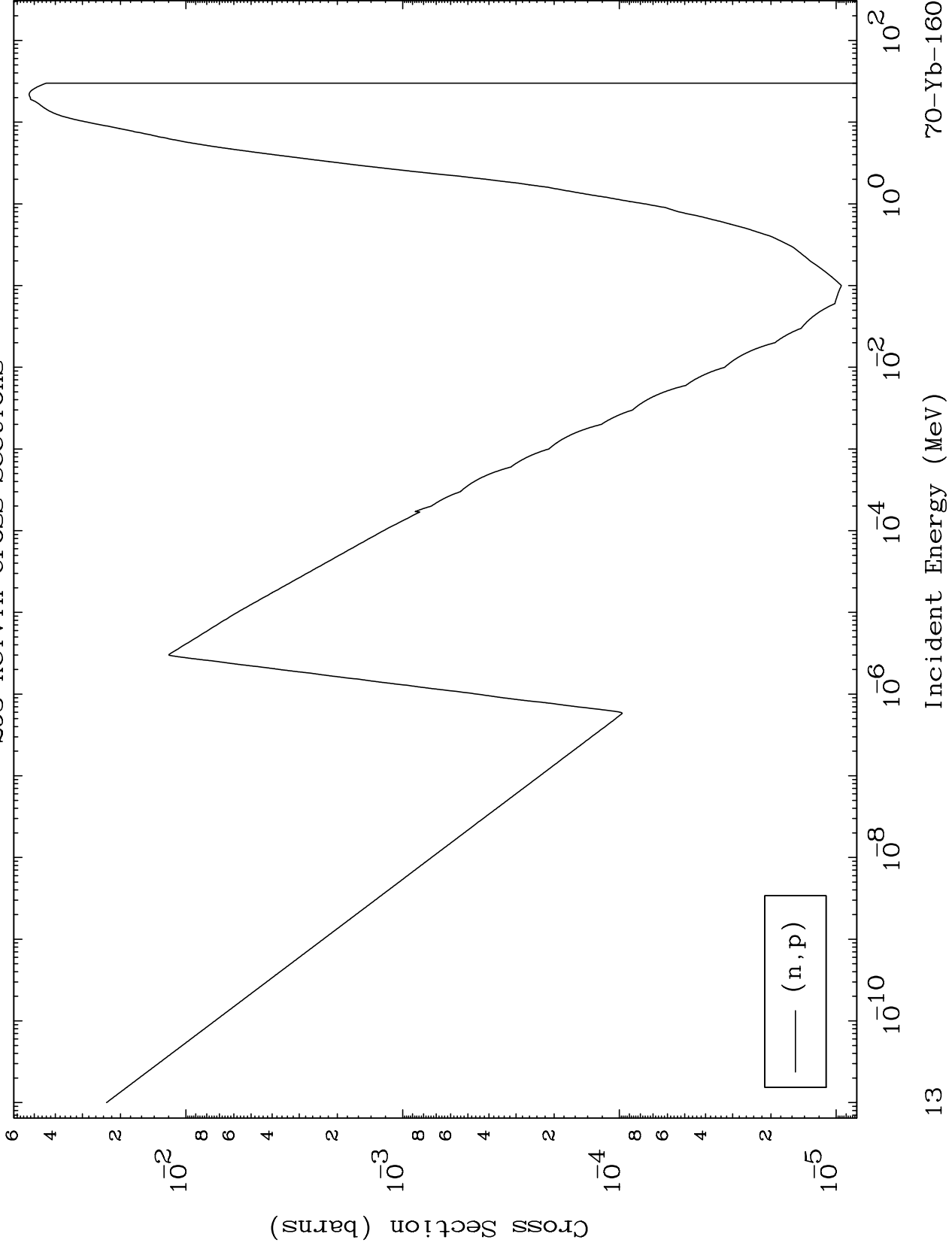




MAT 7001

(n,p) Levels
293 Kelvin Cross Sections

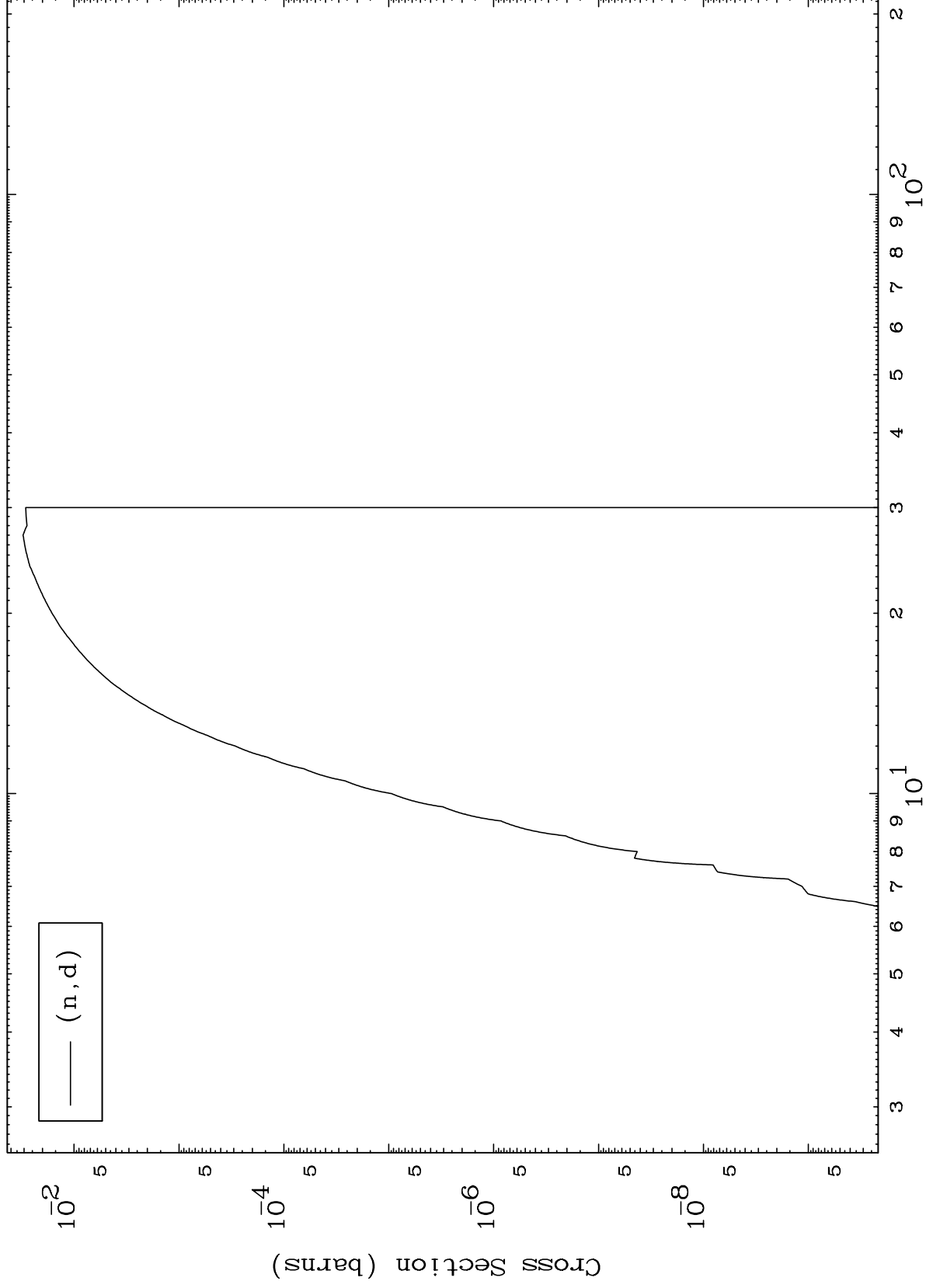
70-Yb-160



MAT 7001

(n,d) Levels
293 Kelvin Cross Sections

70-Yb-160



14

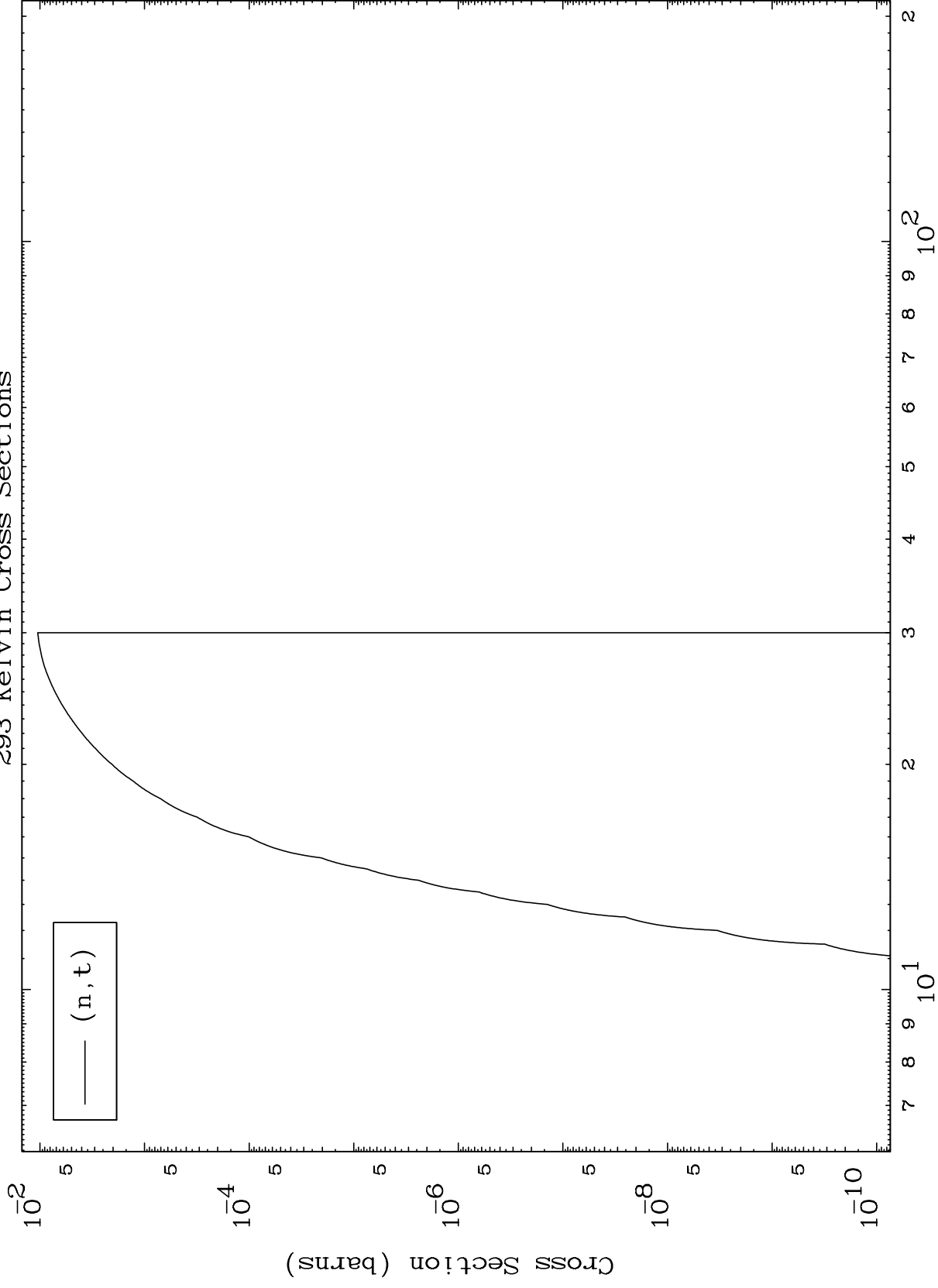
Incident Energy (MeV)

70-Yb-160

MAT 7001

(n,t) Levels
293 Kelvin Cross Sections

70-Yb-160



15

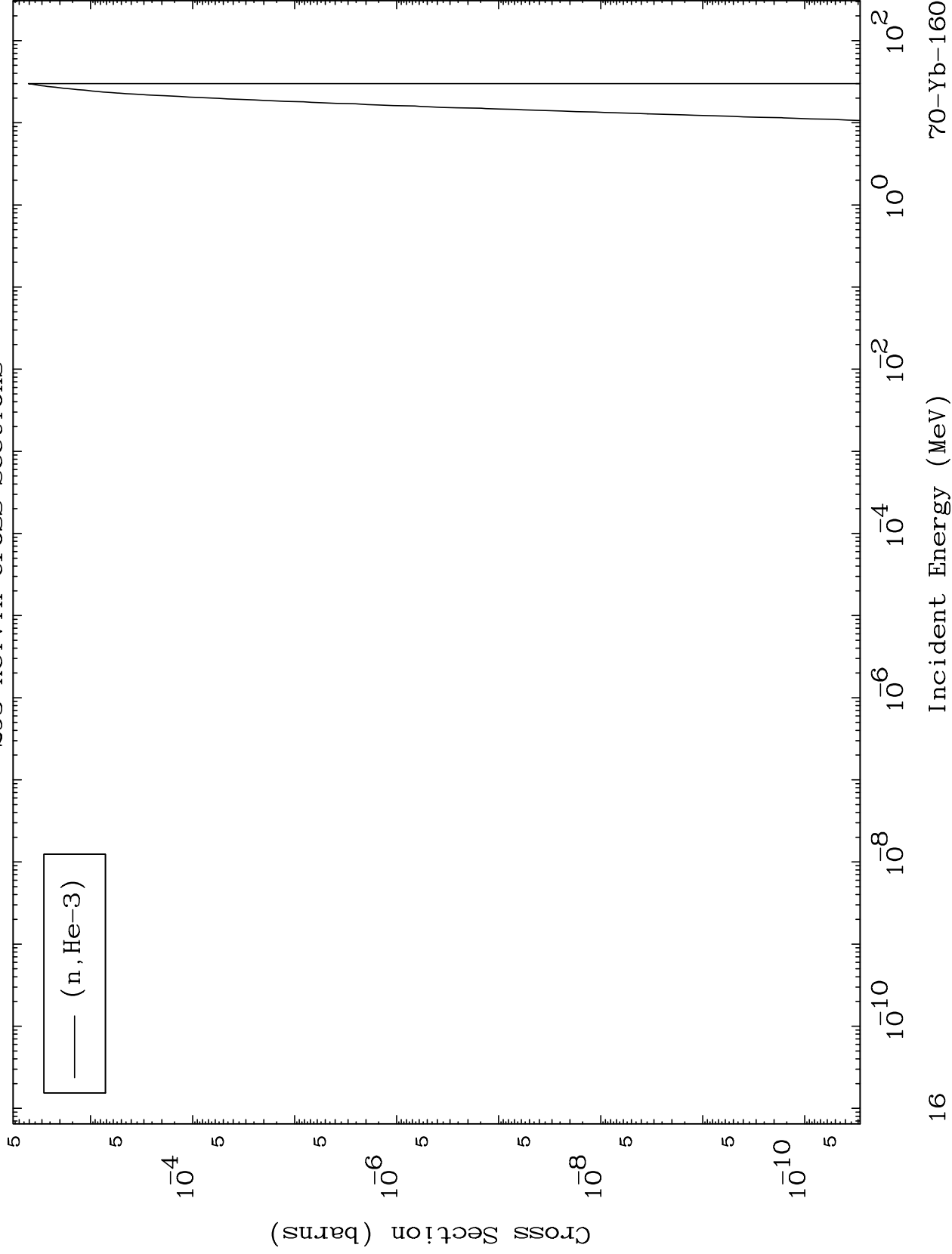
Incident Energy (MeV)

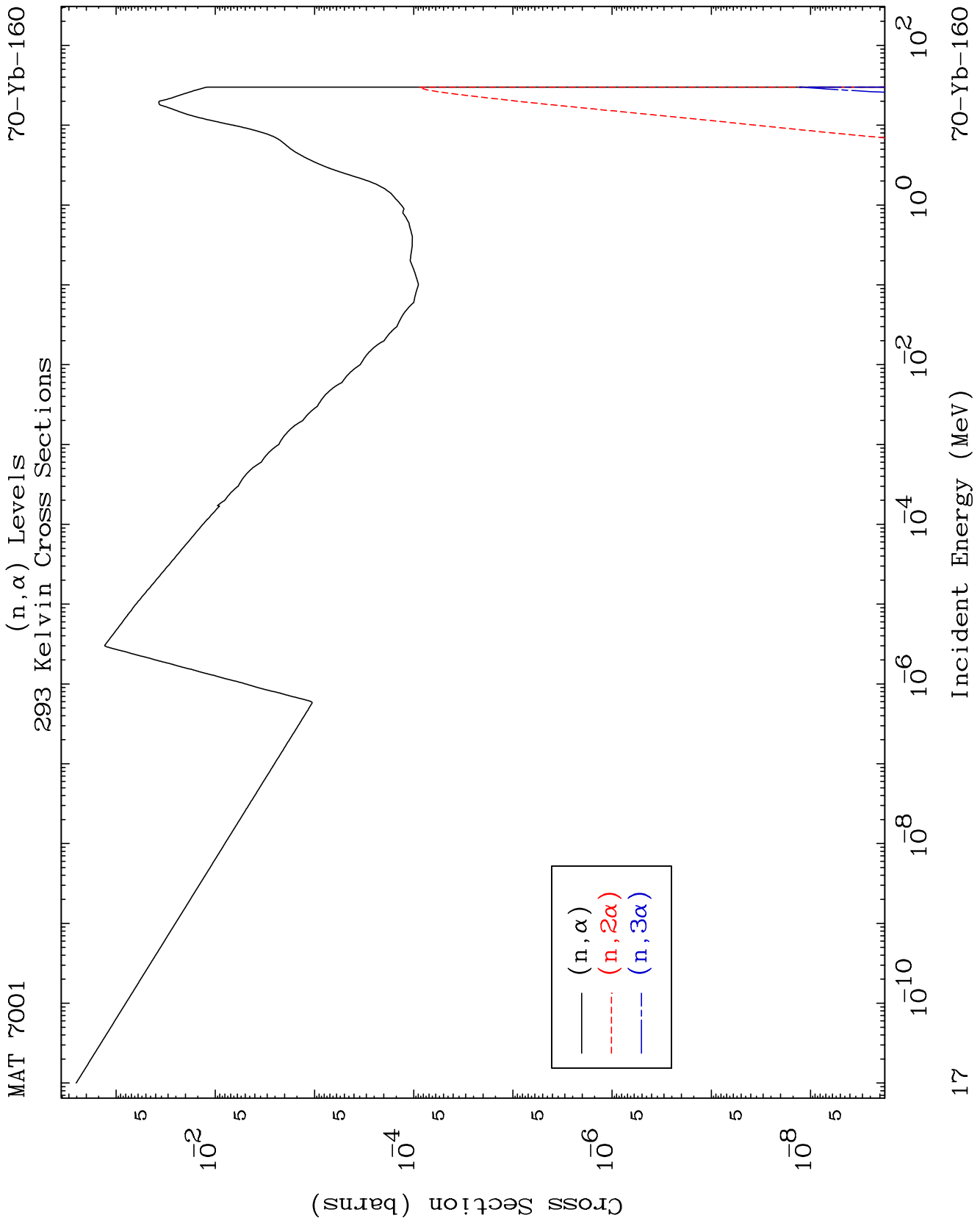
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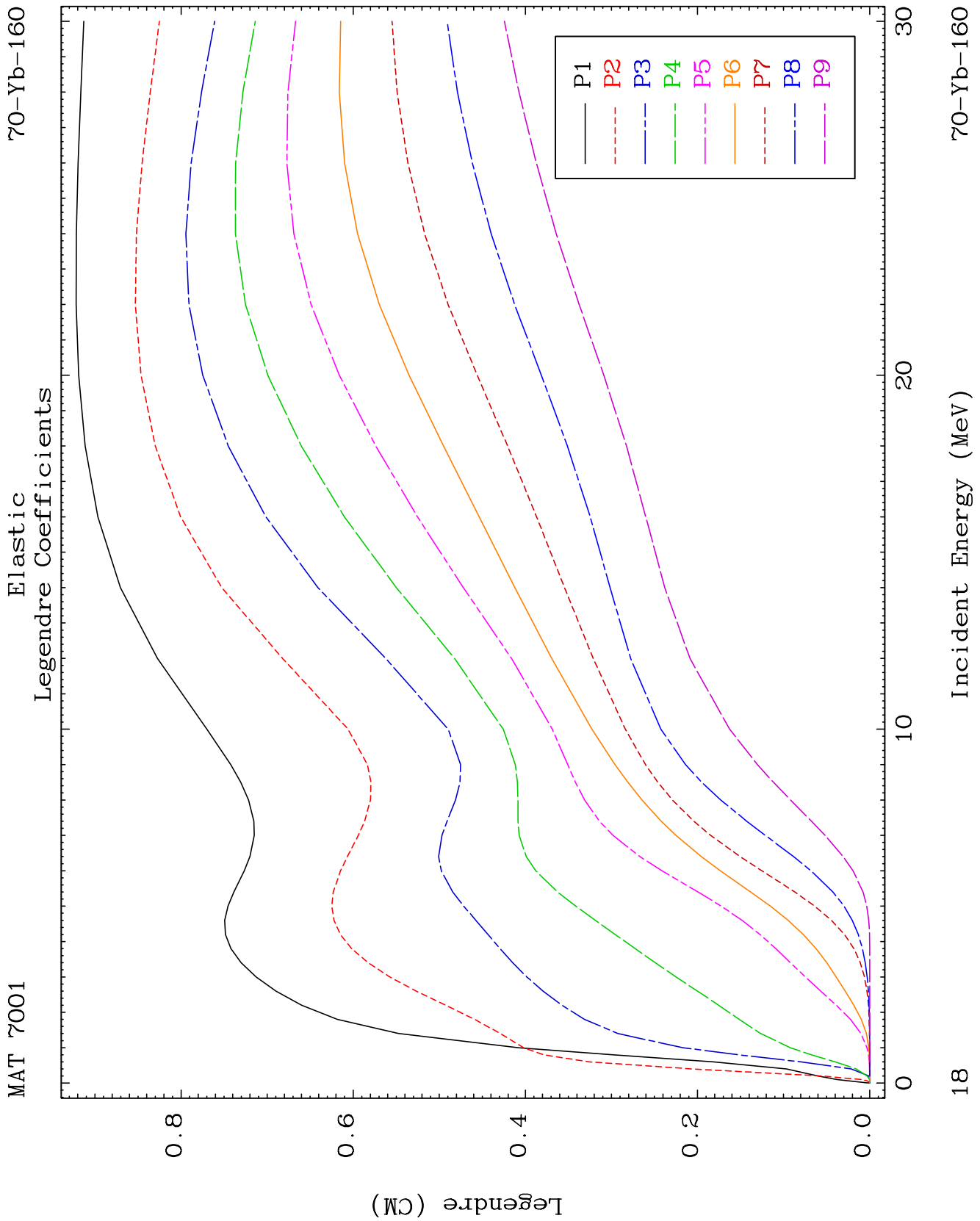
MAT 7001

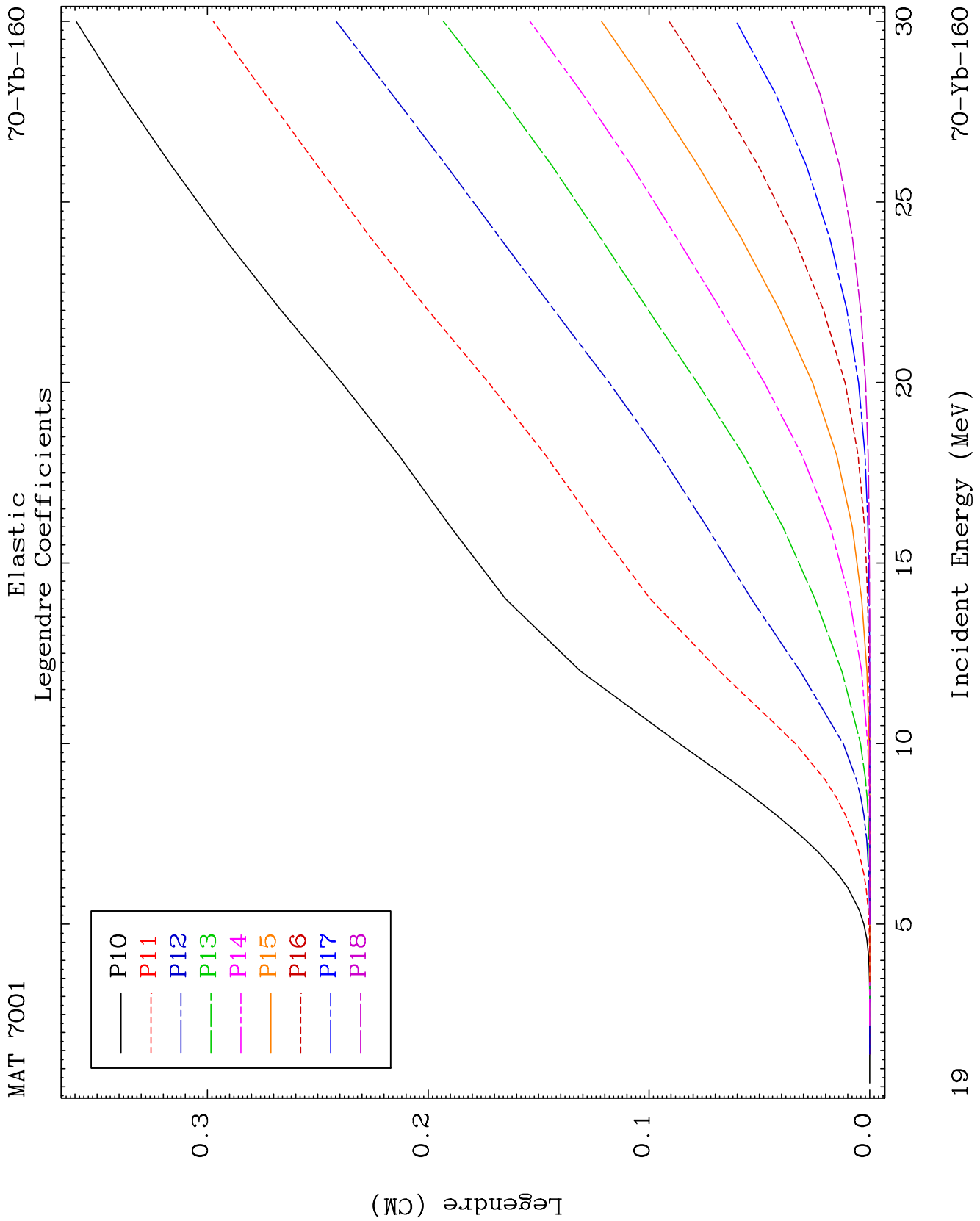
(n,He3) Levels
293 Kelvin Cross Sections

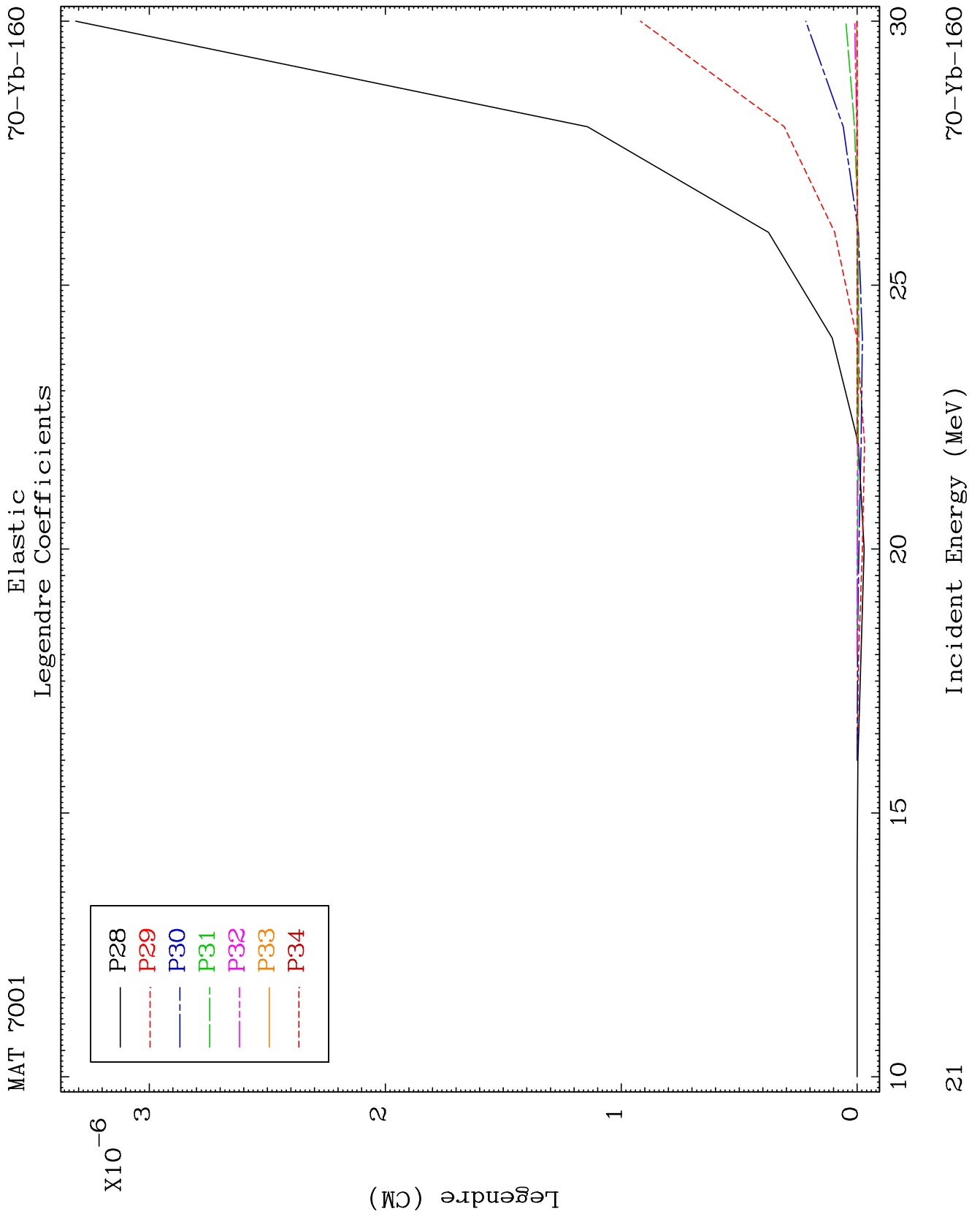
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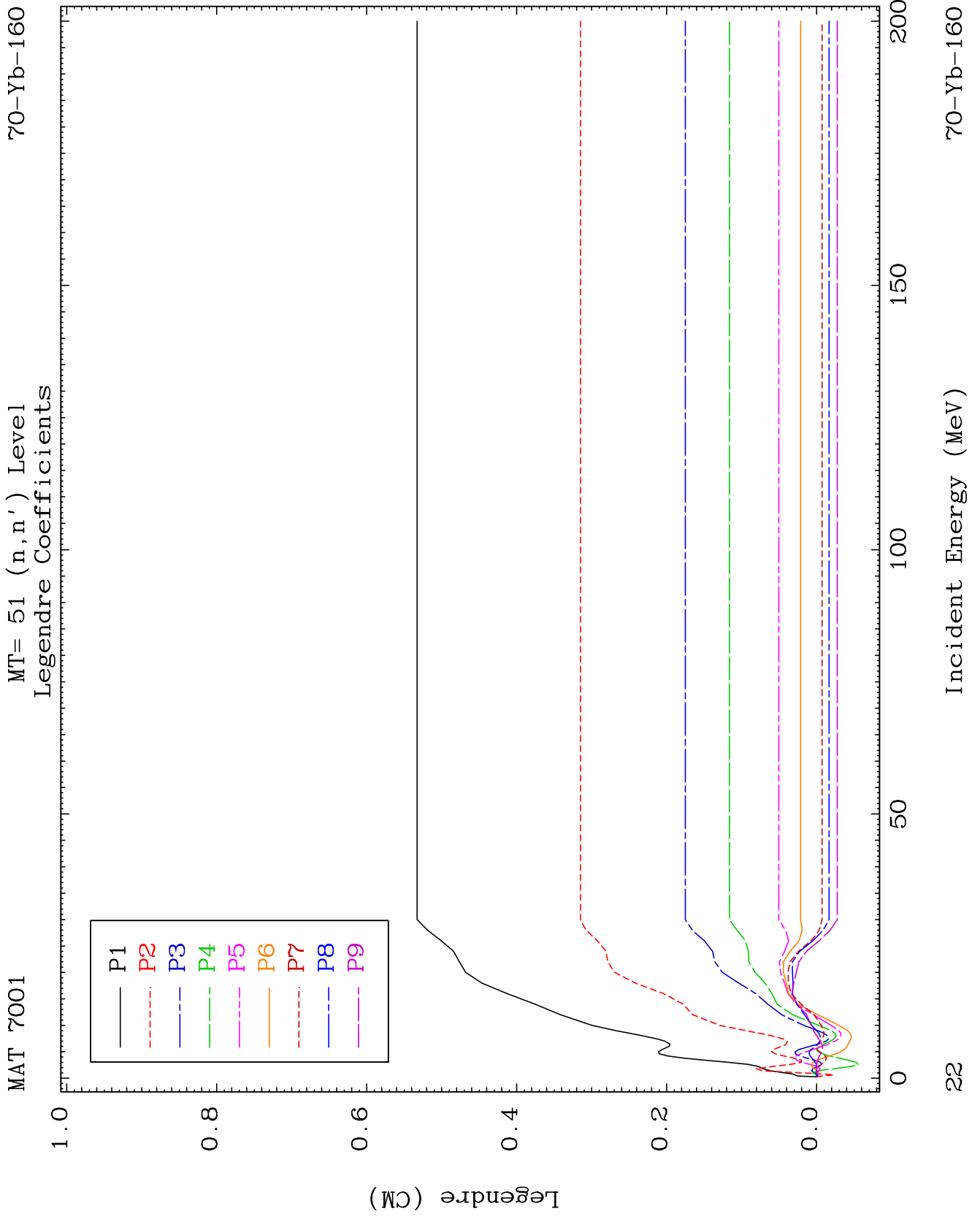


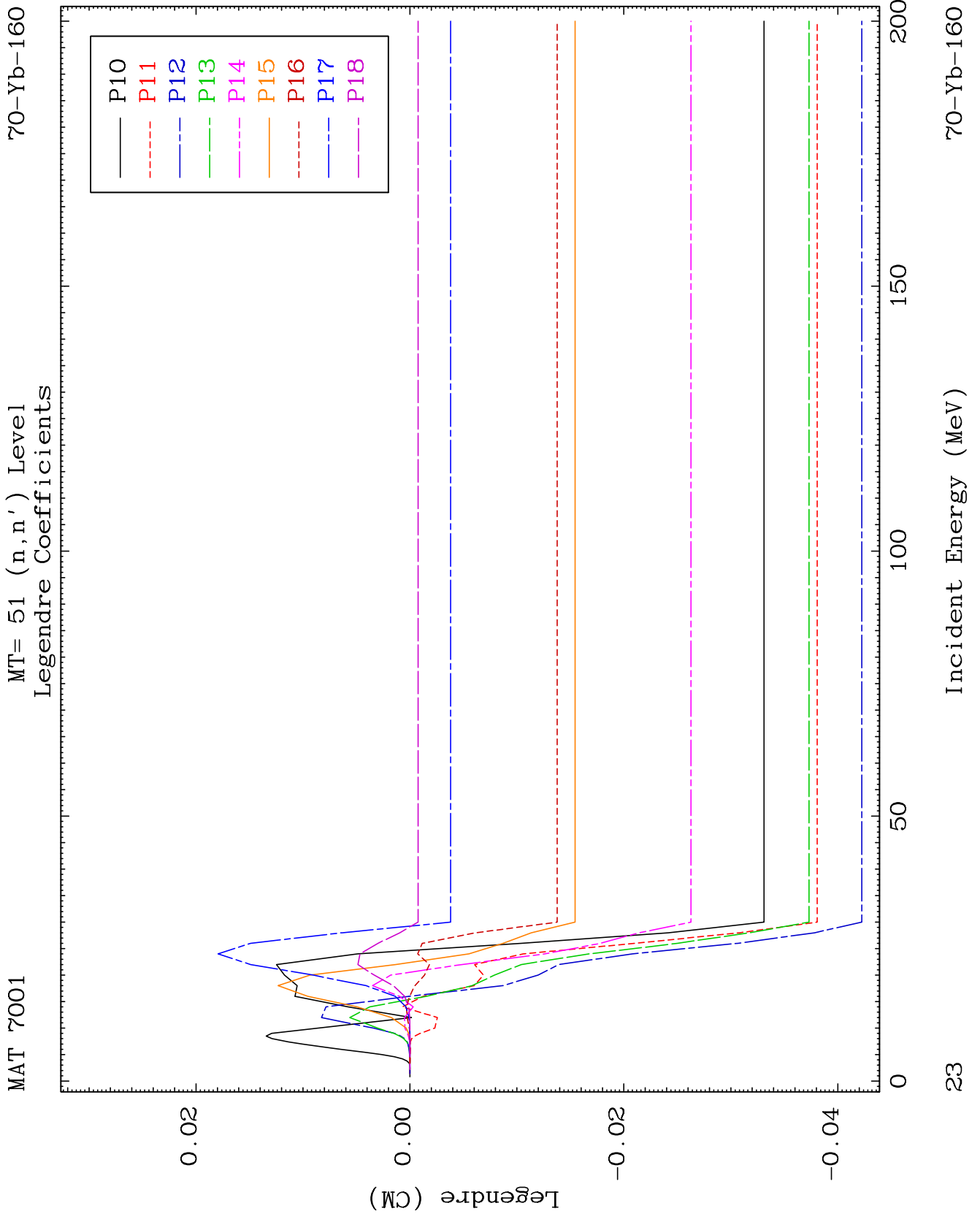


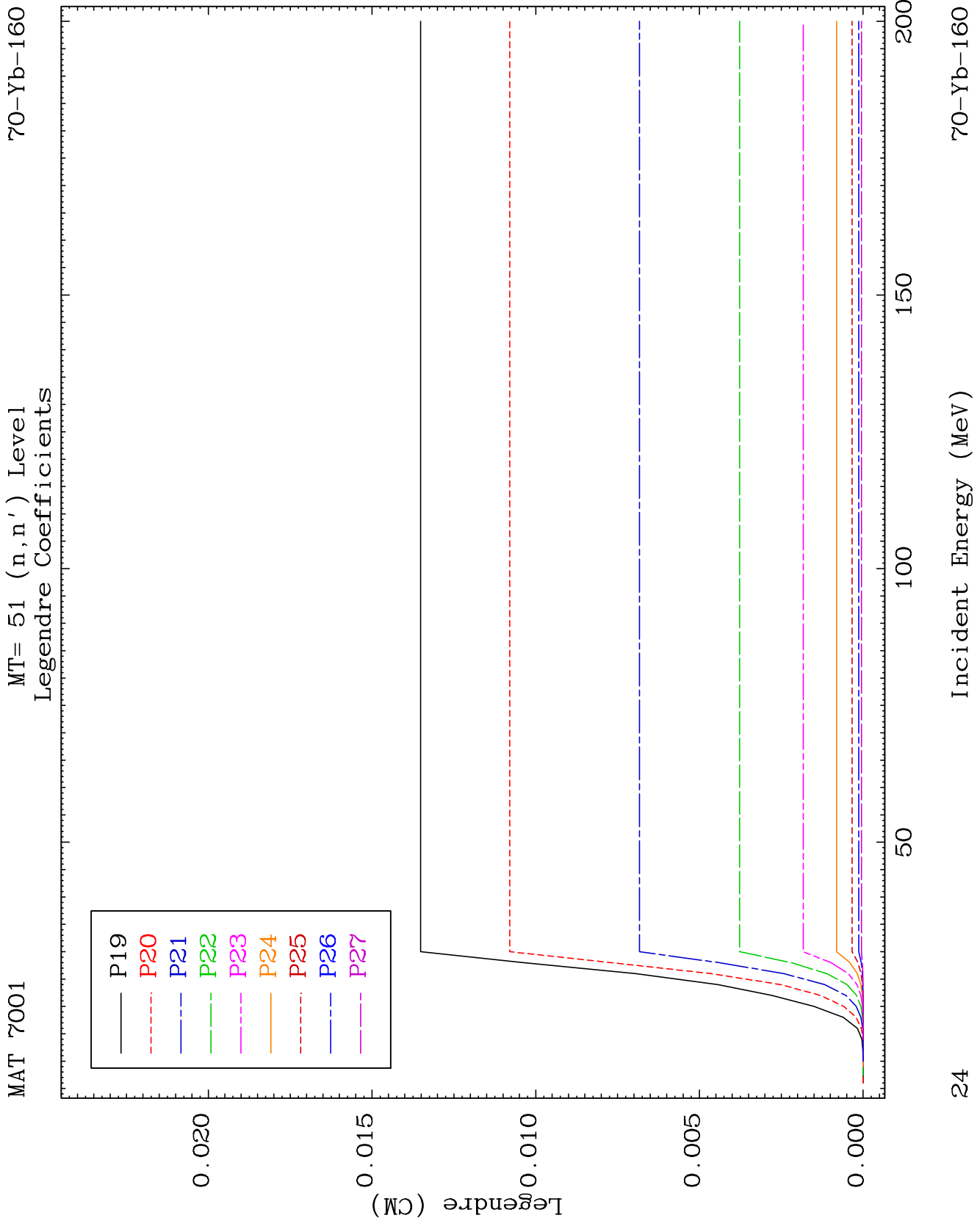


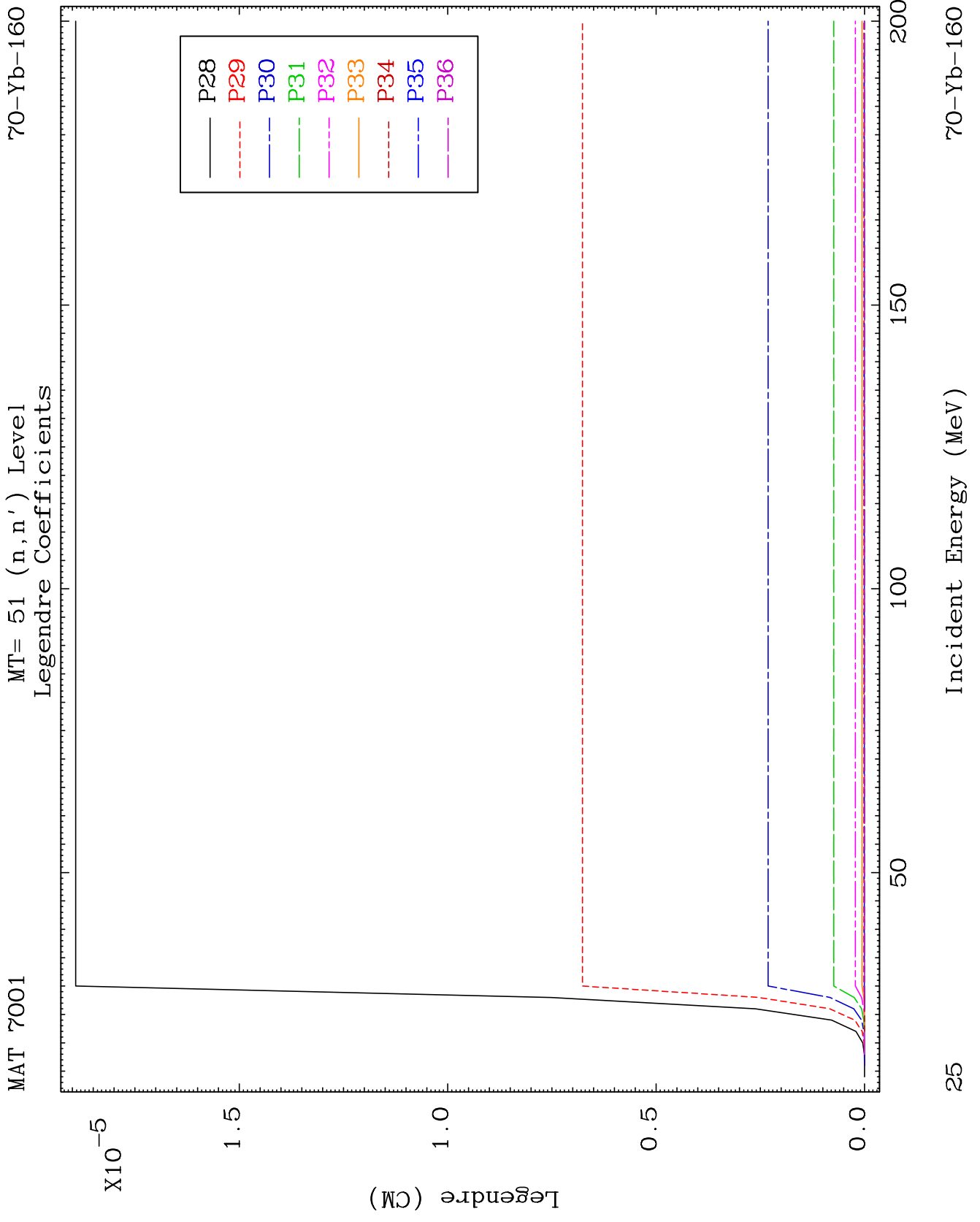


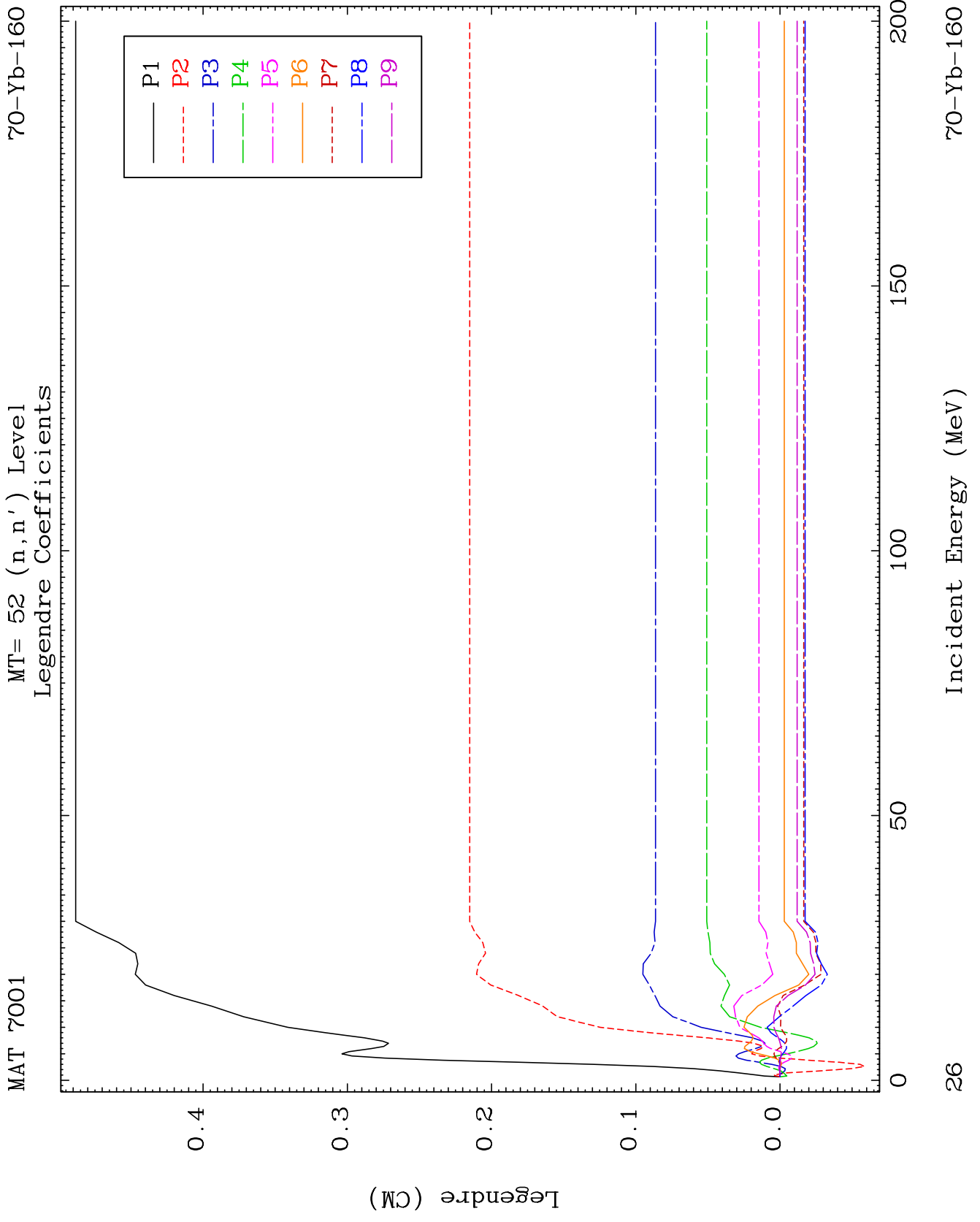


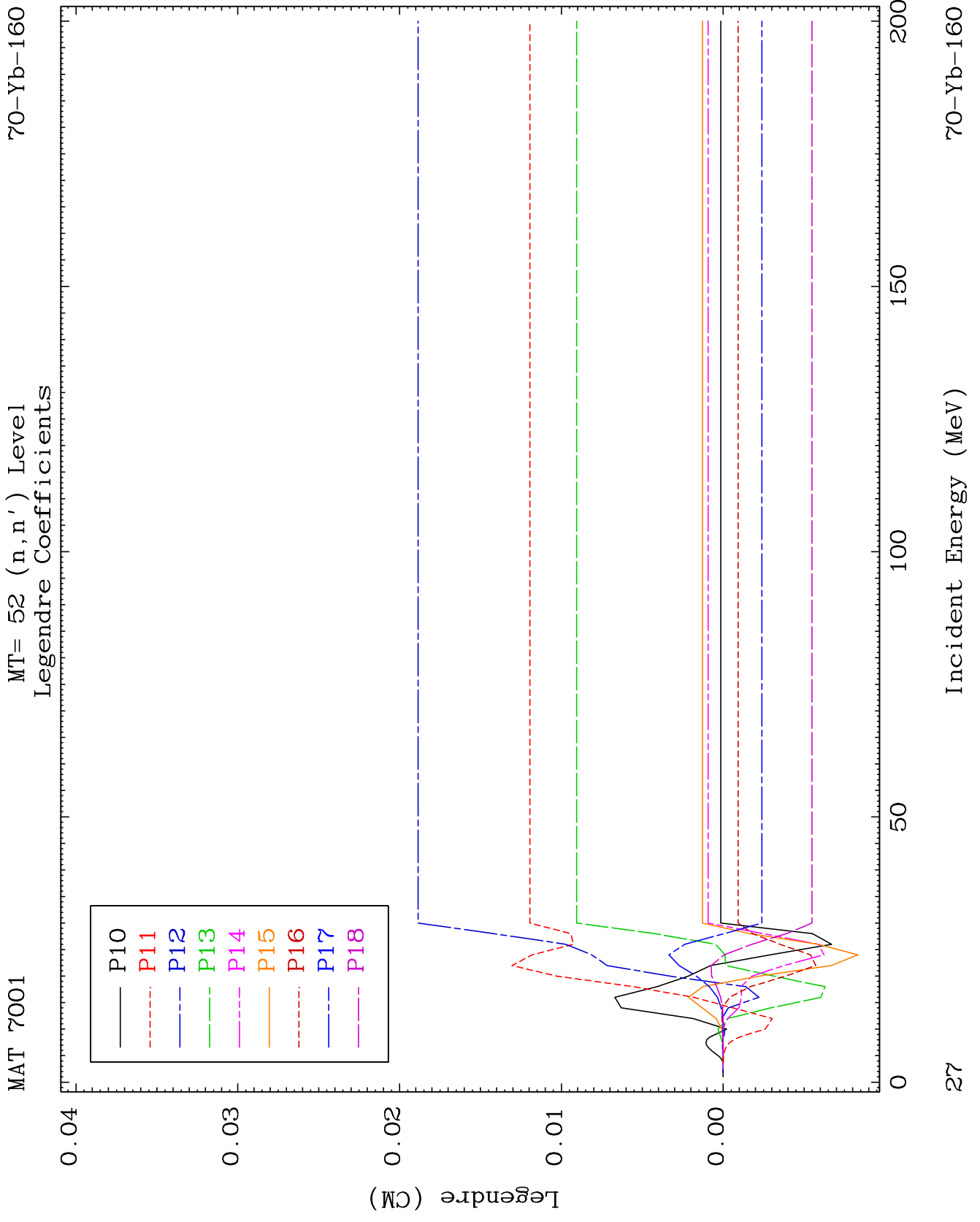








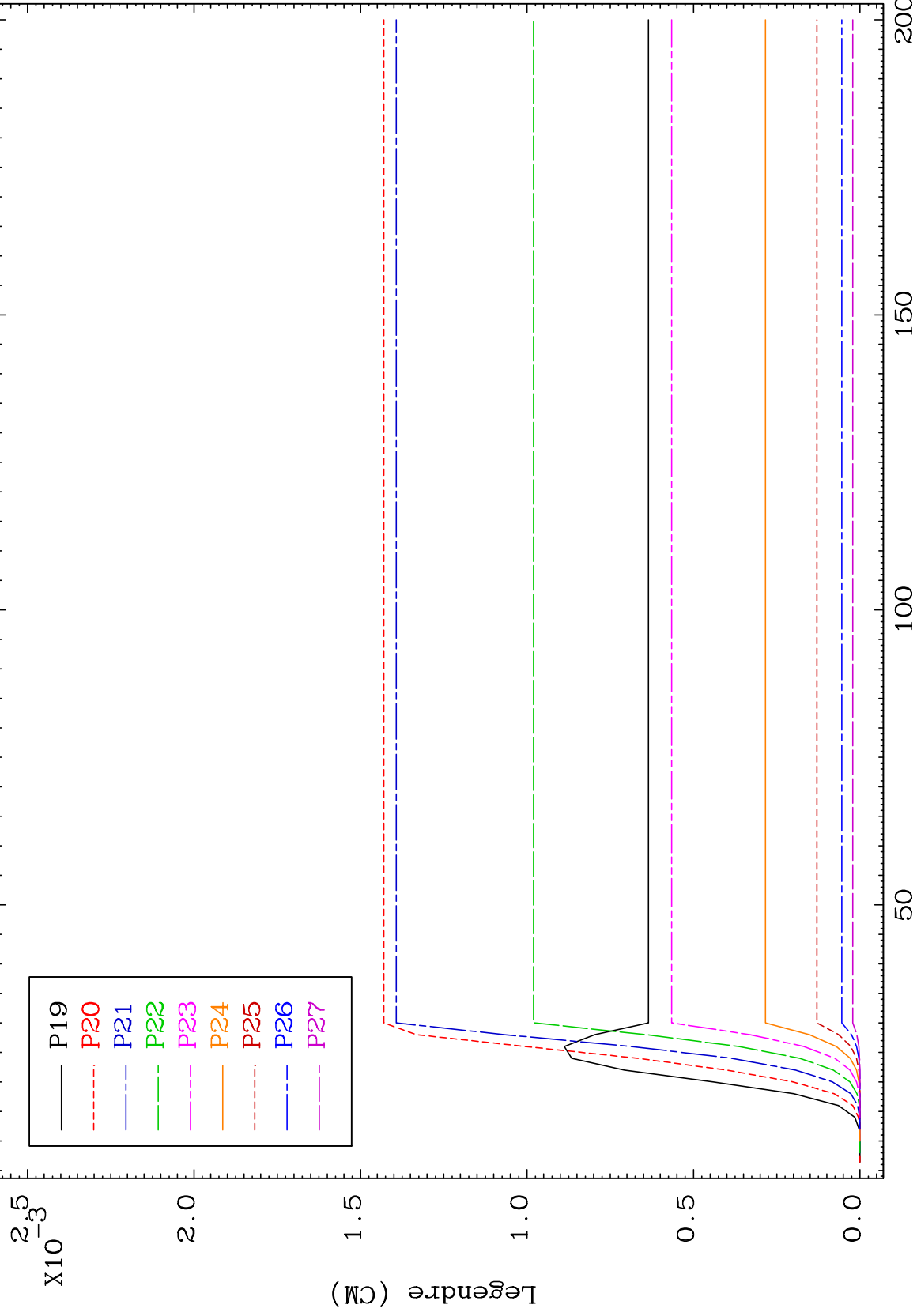




MAT 7001

MT= 52 (n,n') Level
Legendre Coefficients

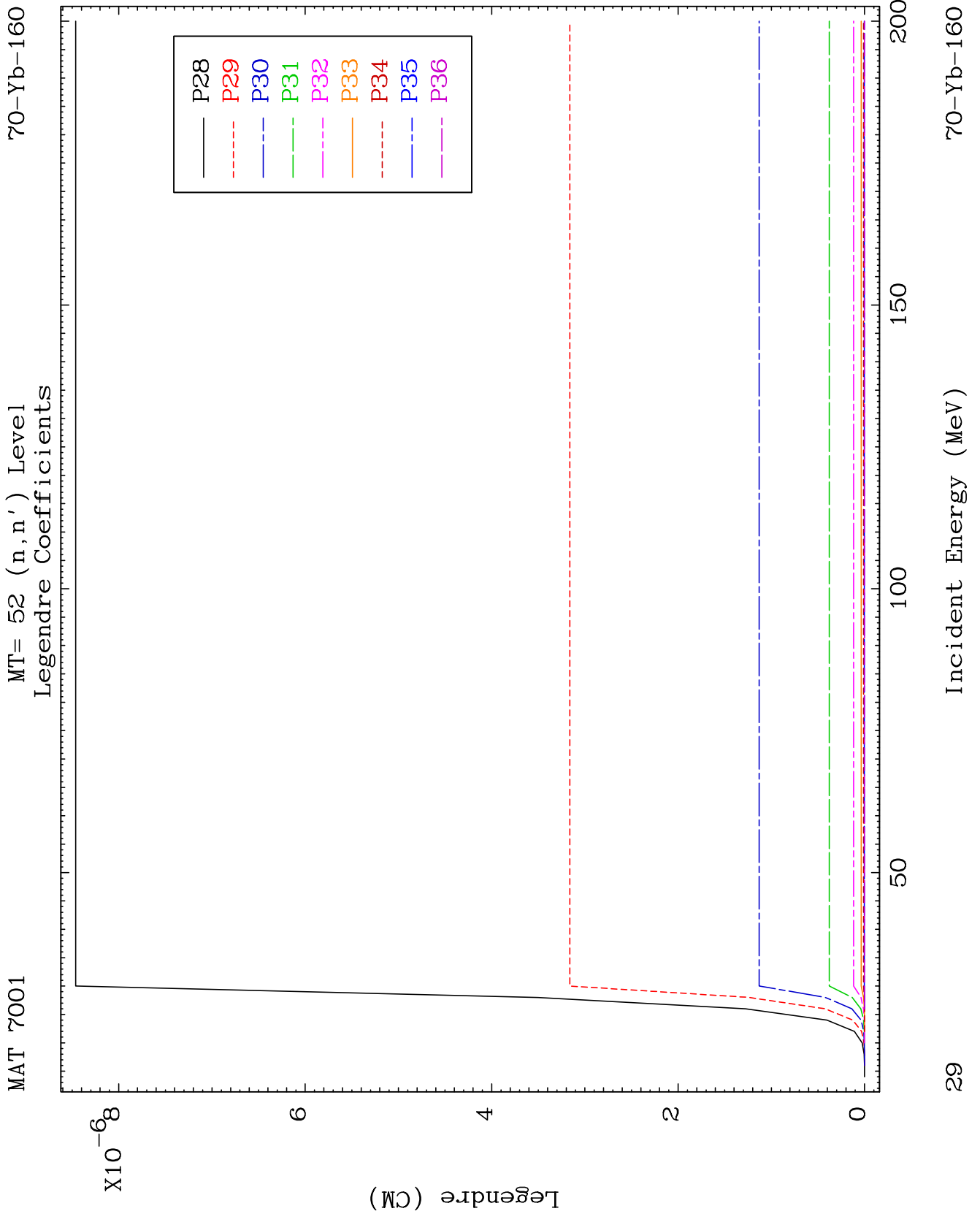
70-Yb-160

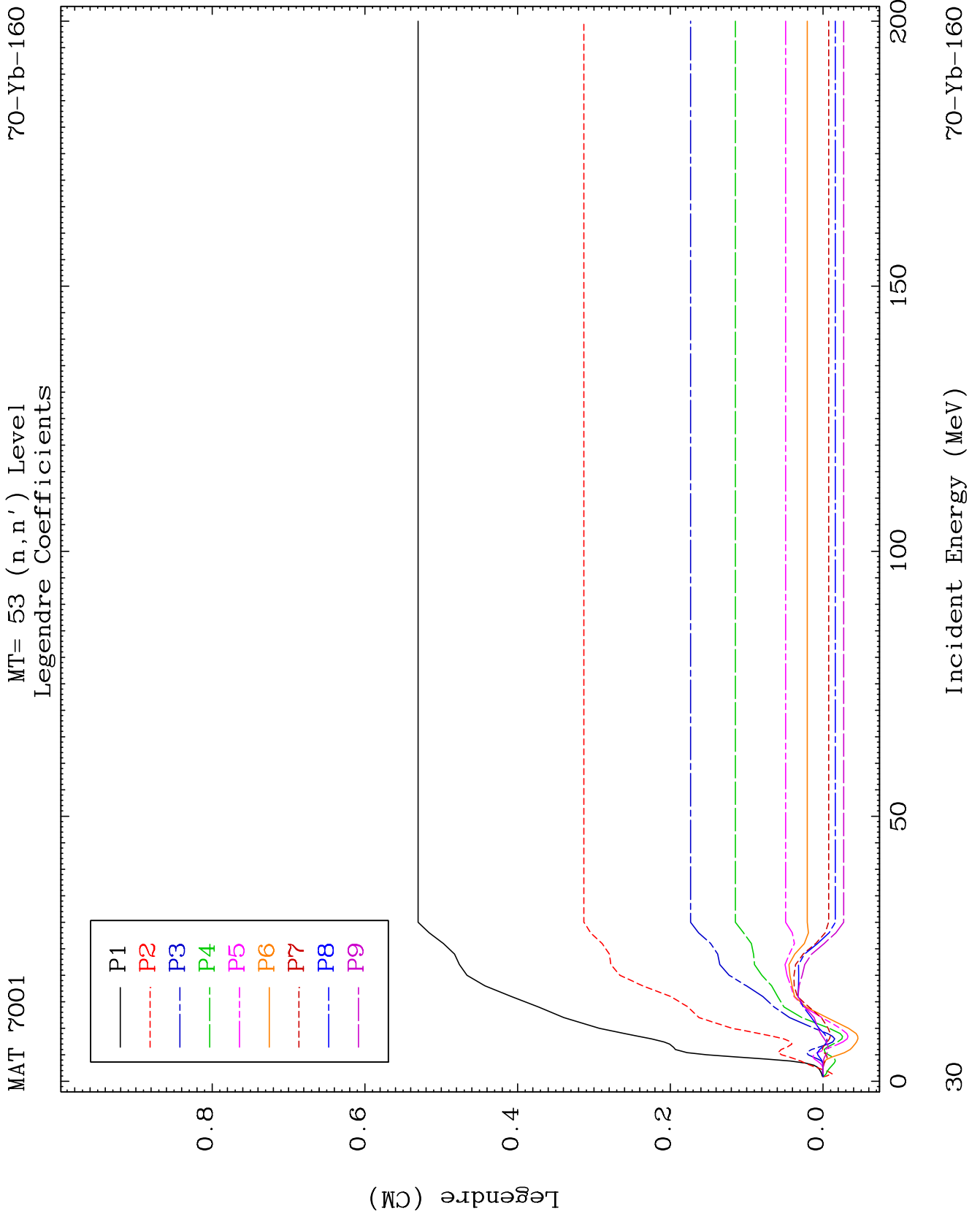


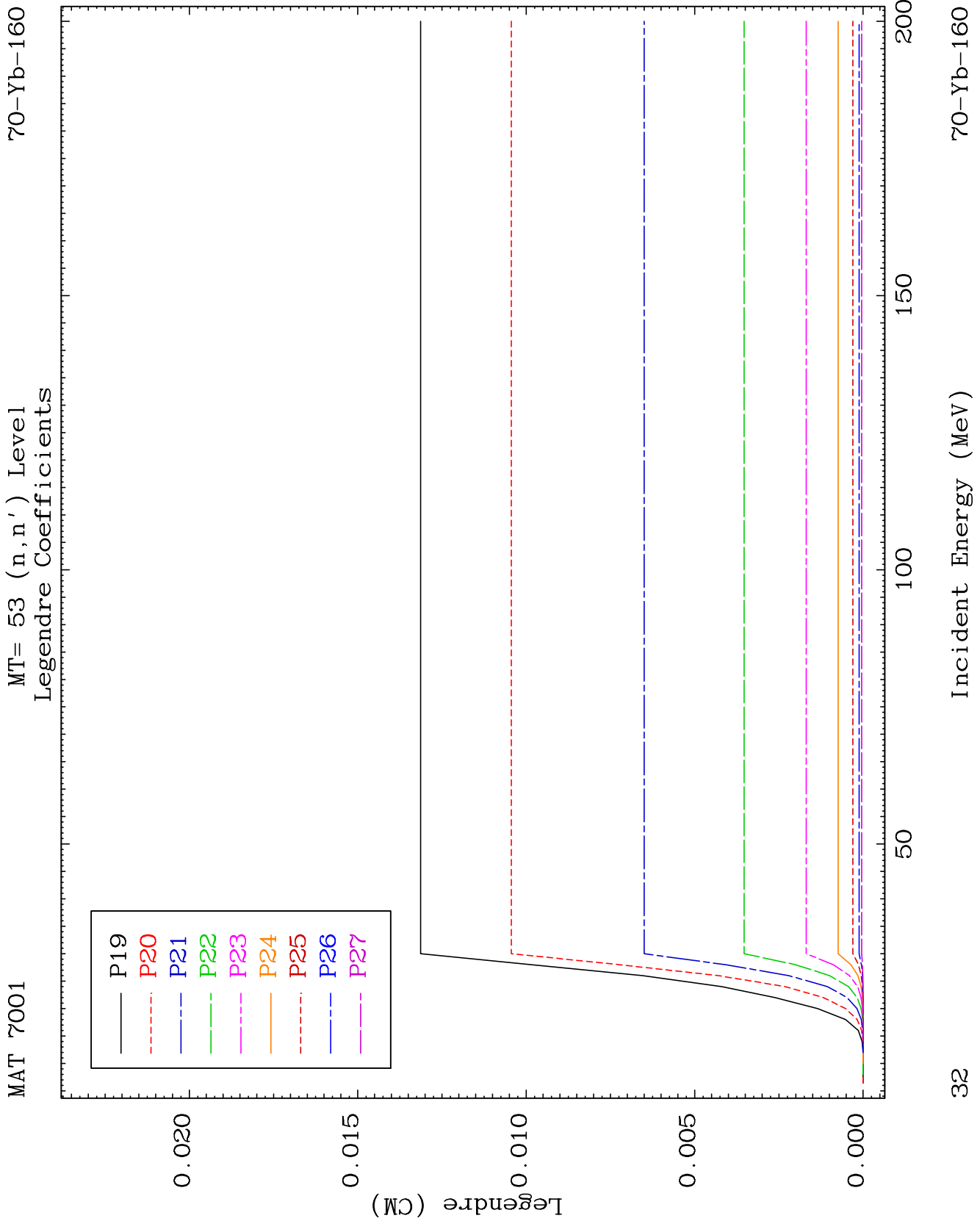
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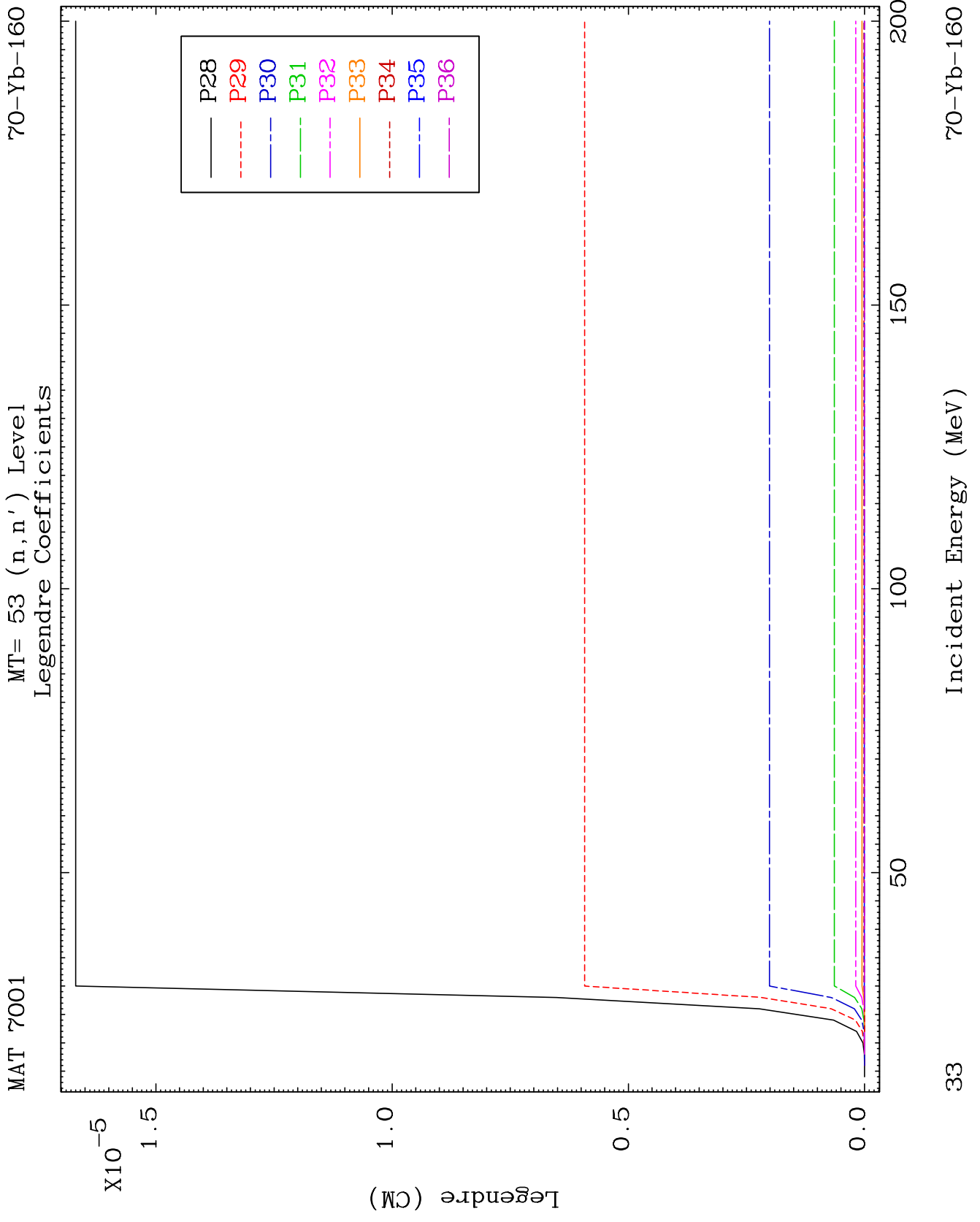
Incident Energy (MeV)

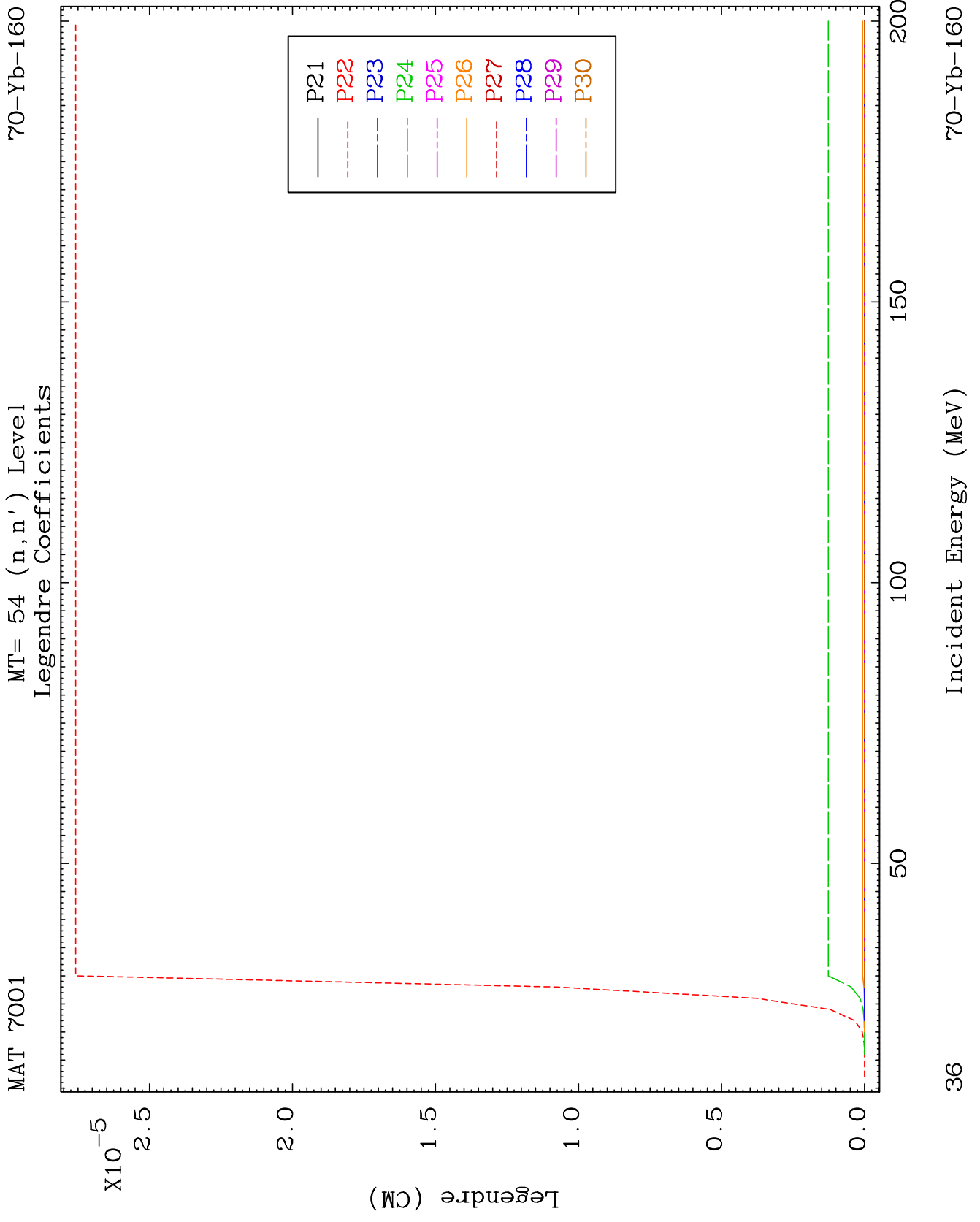
70-Yb-160

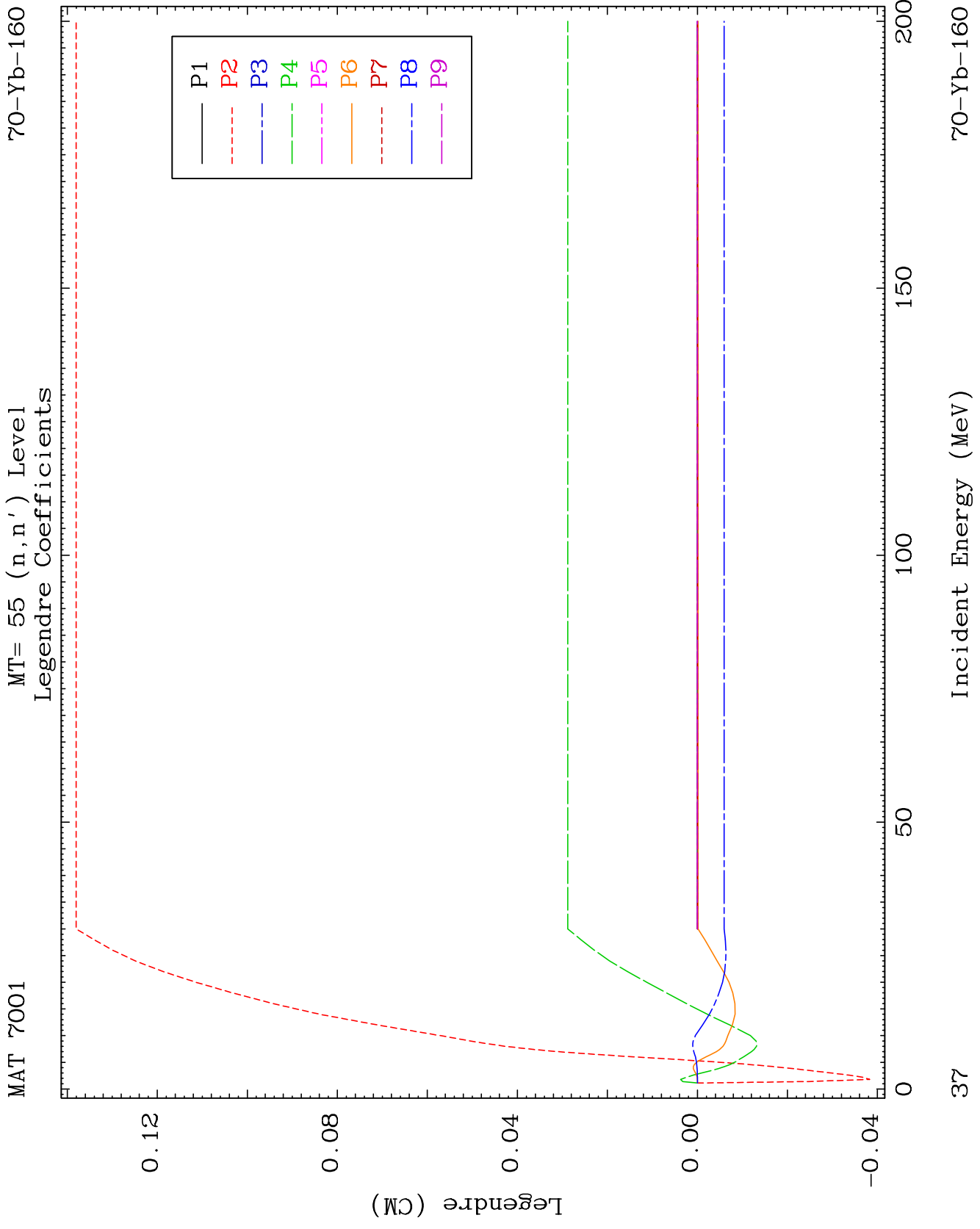


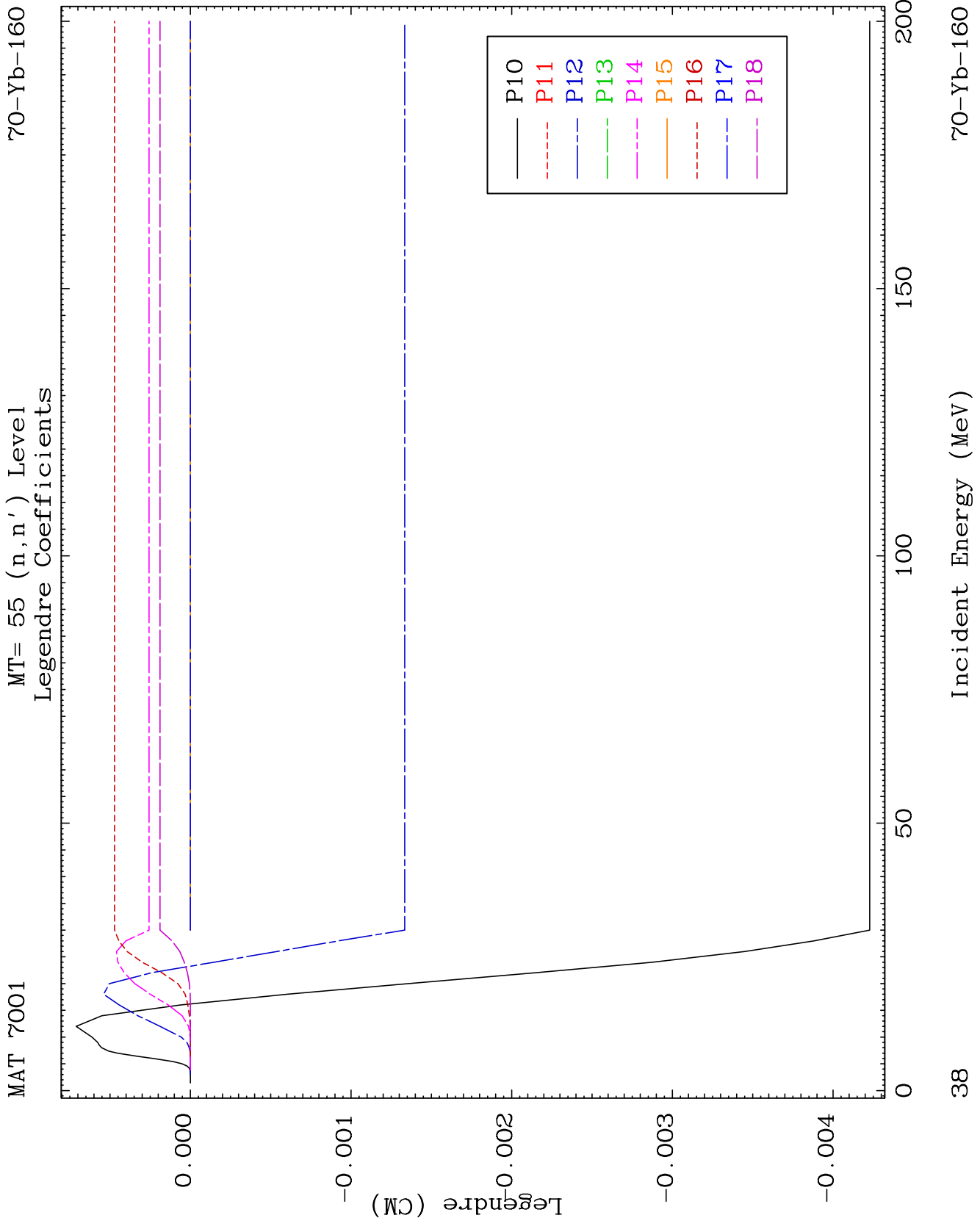


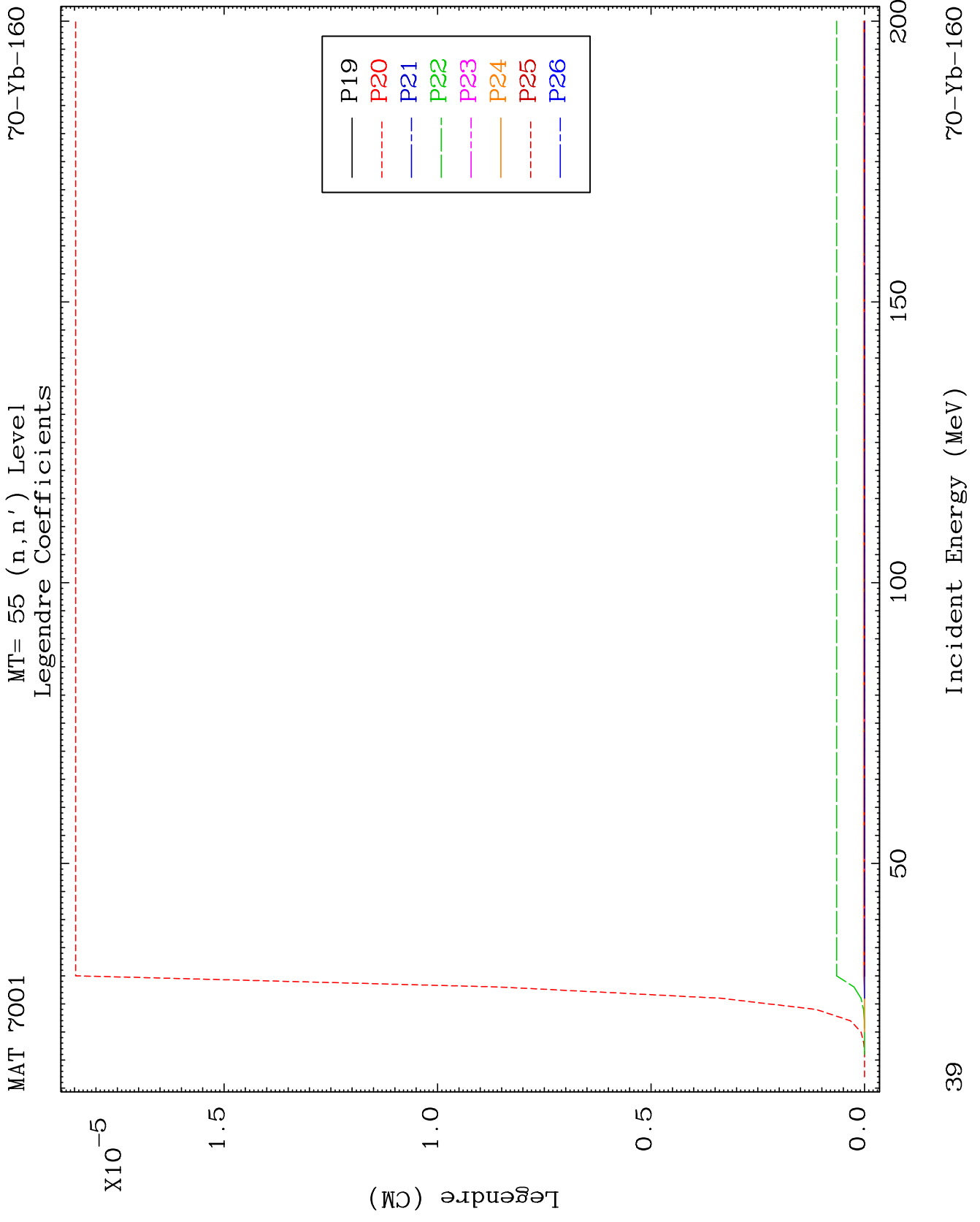


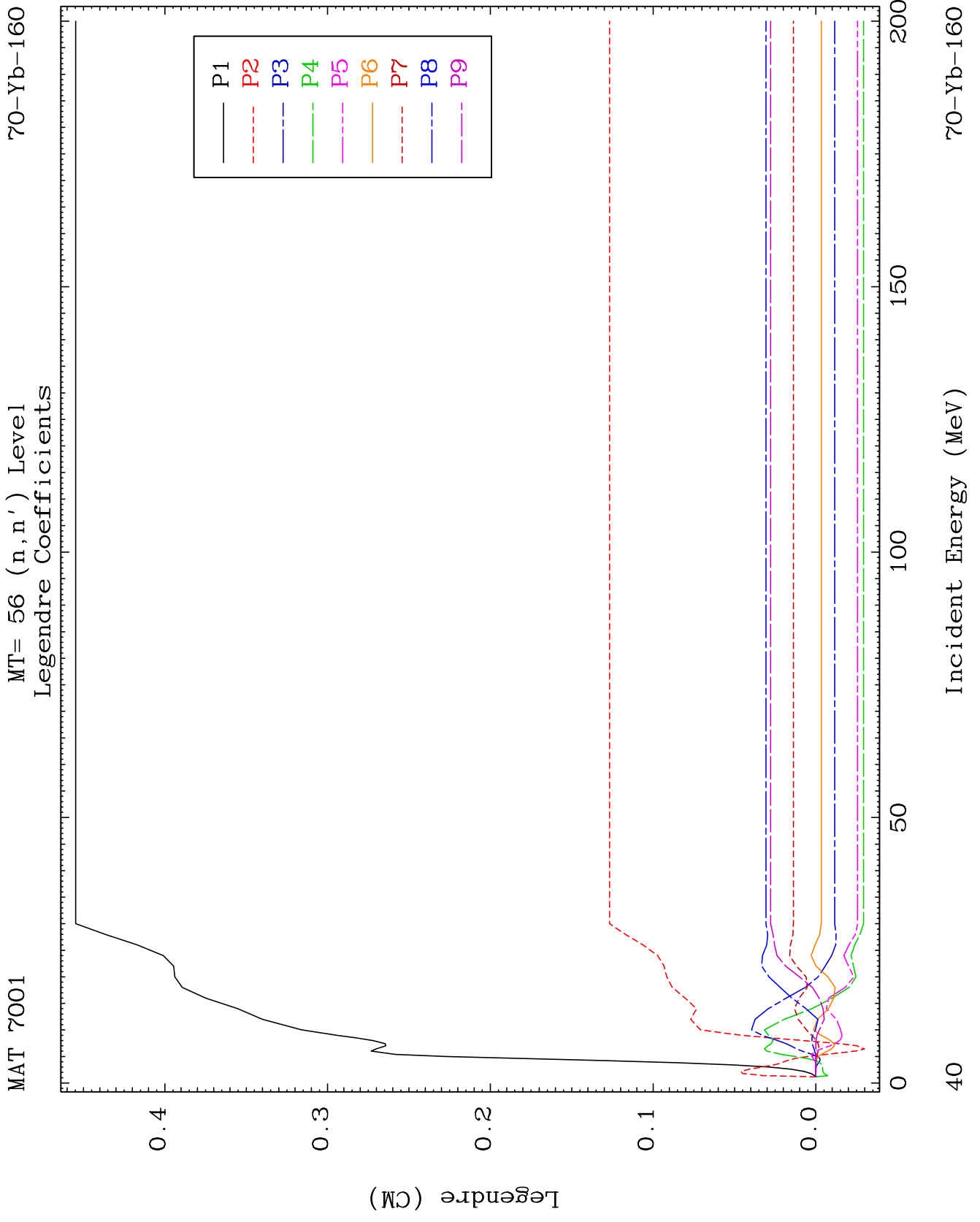


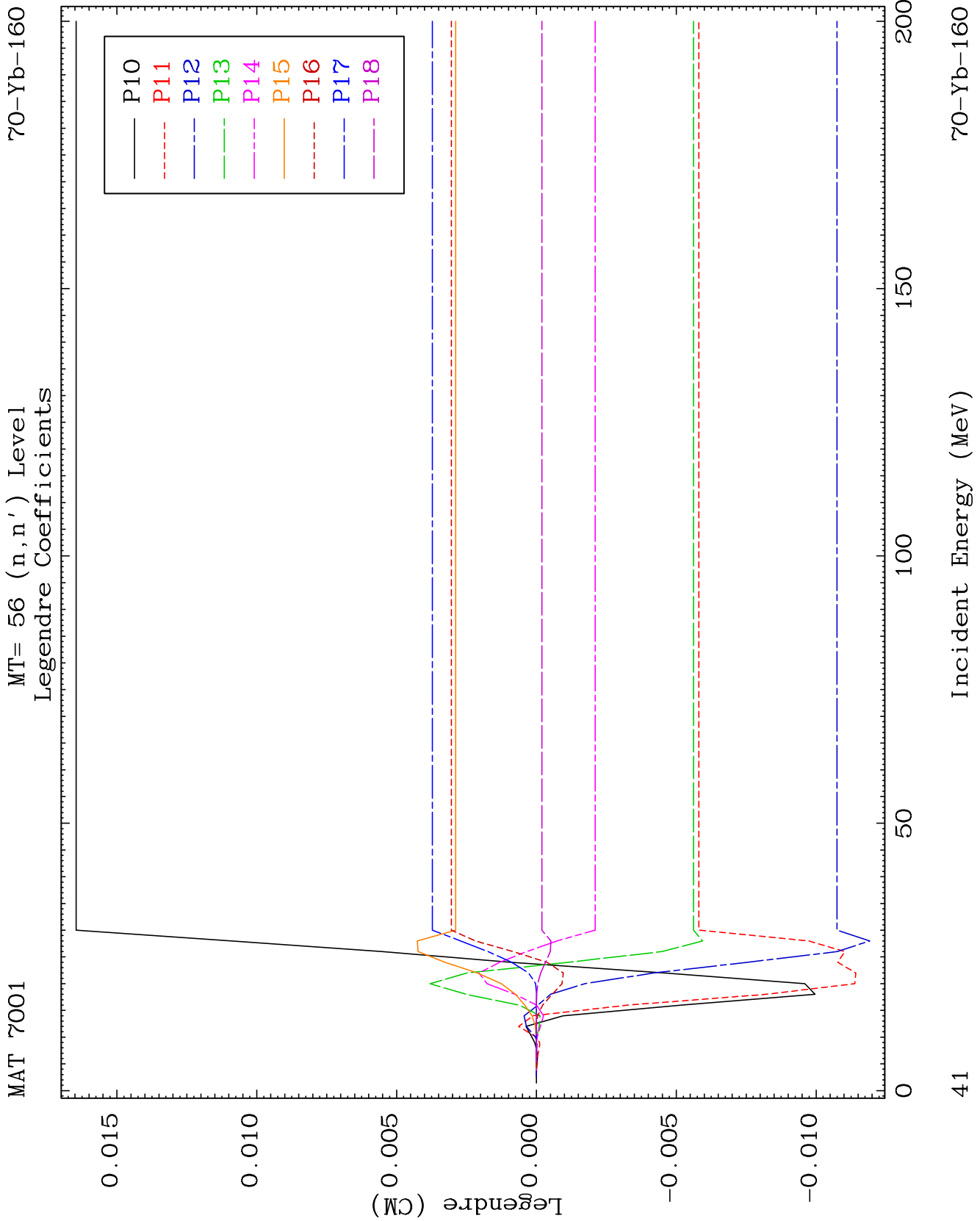


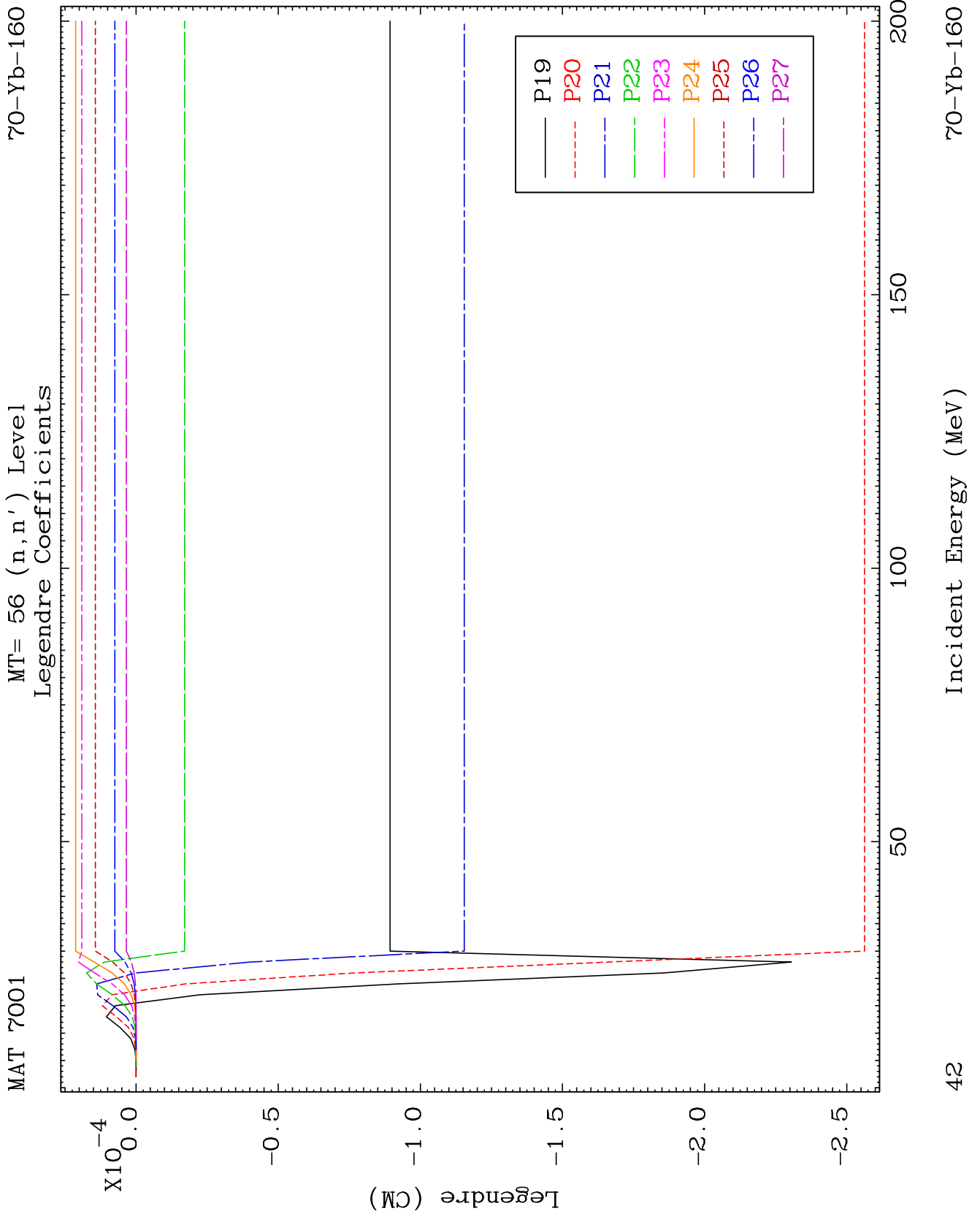


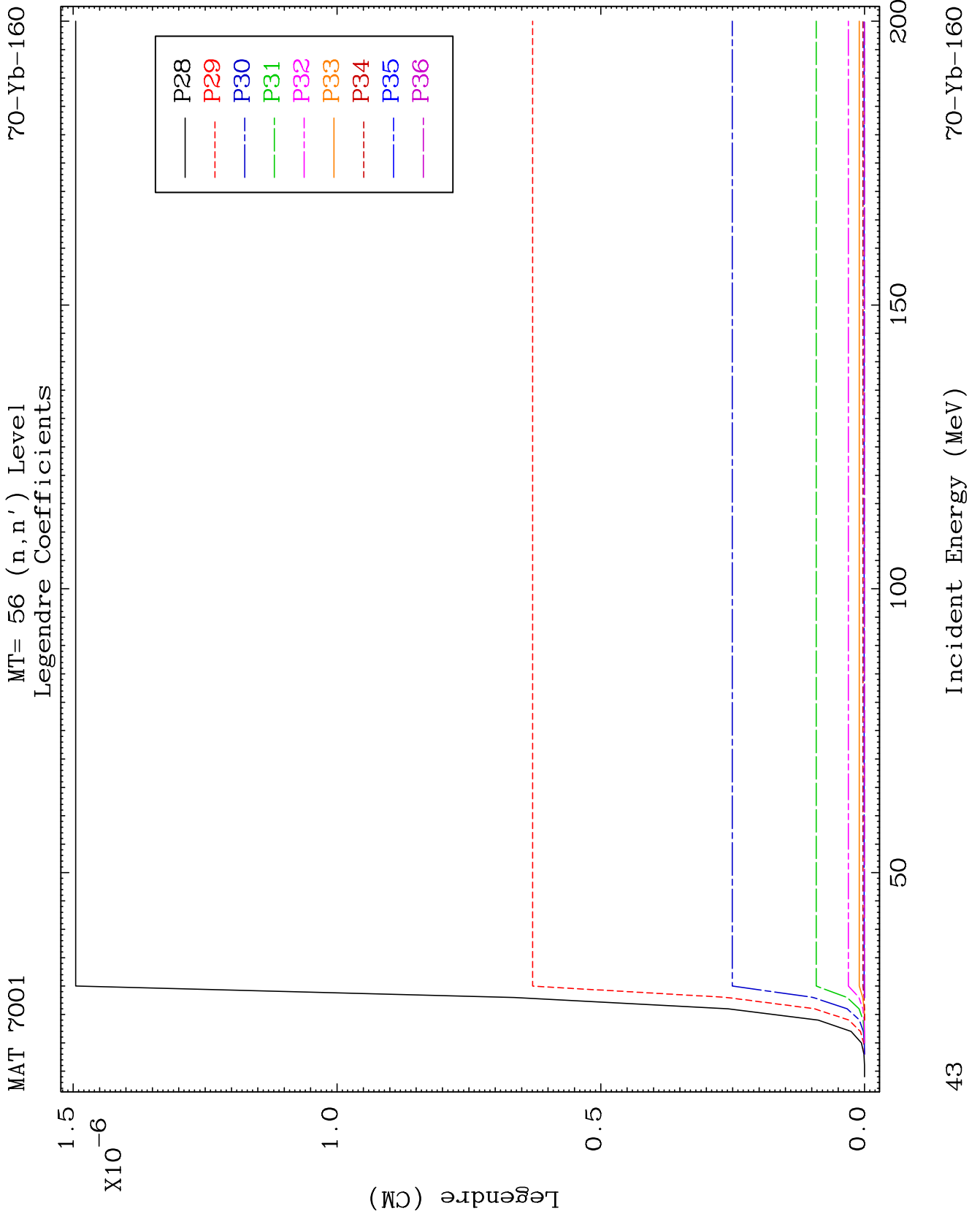


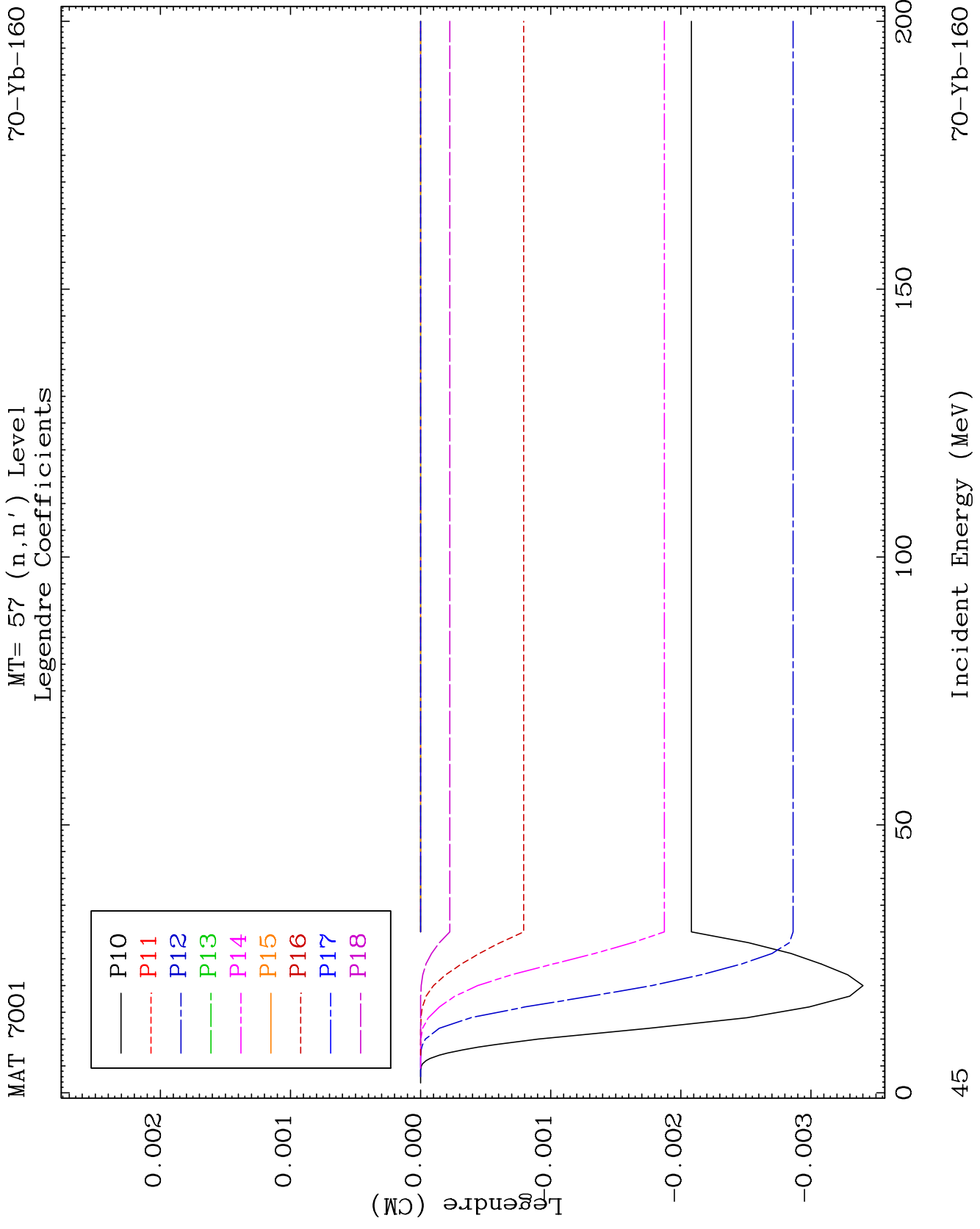


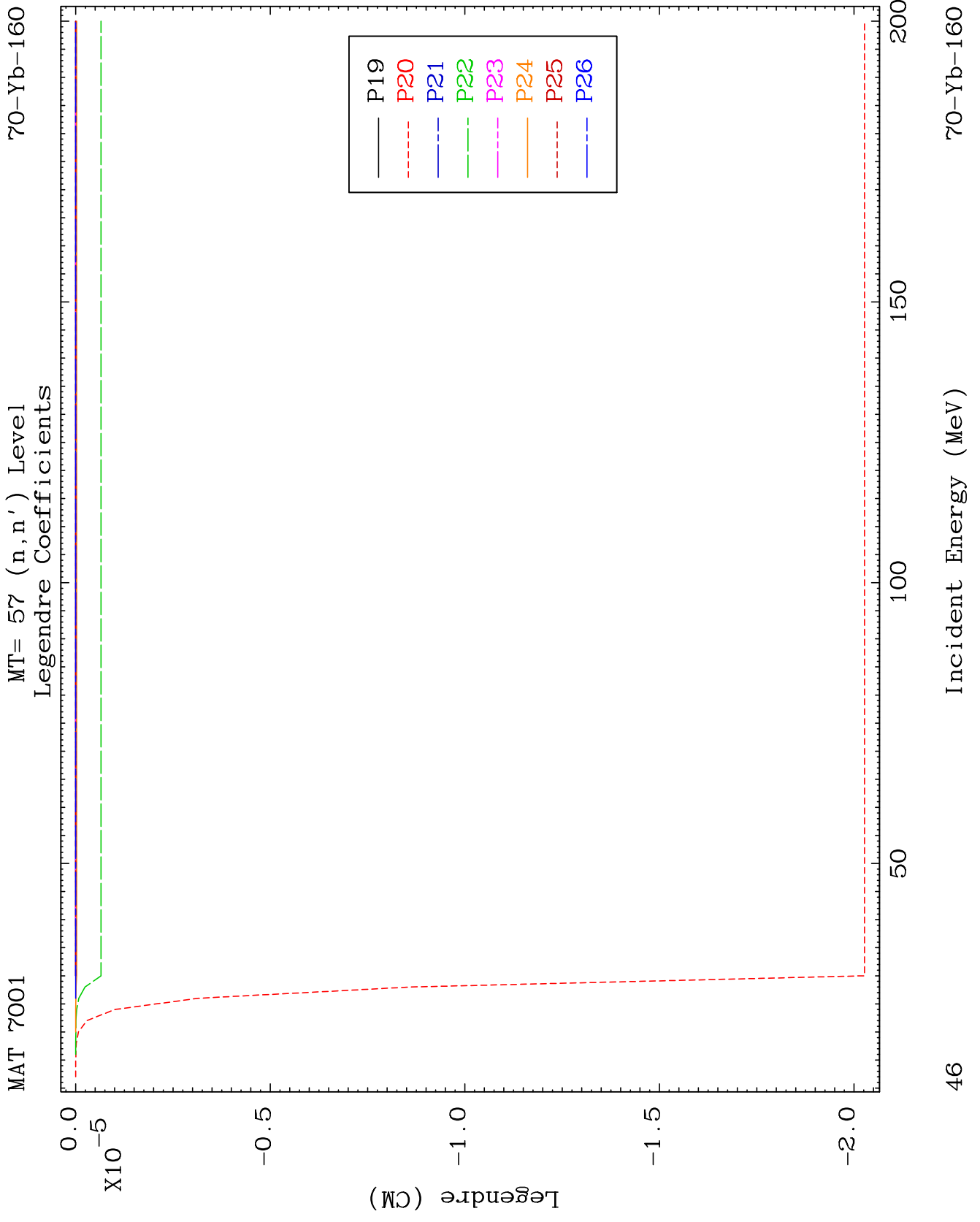


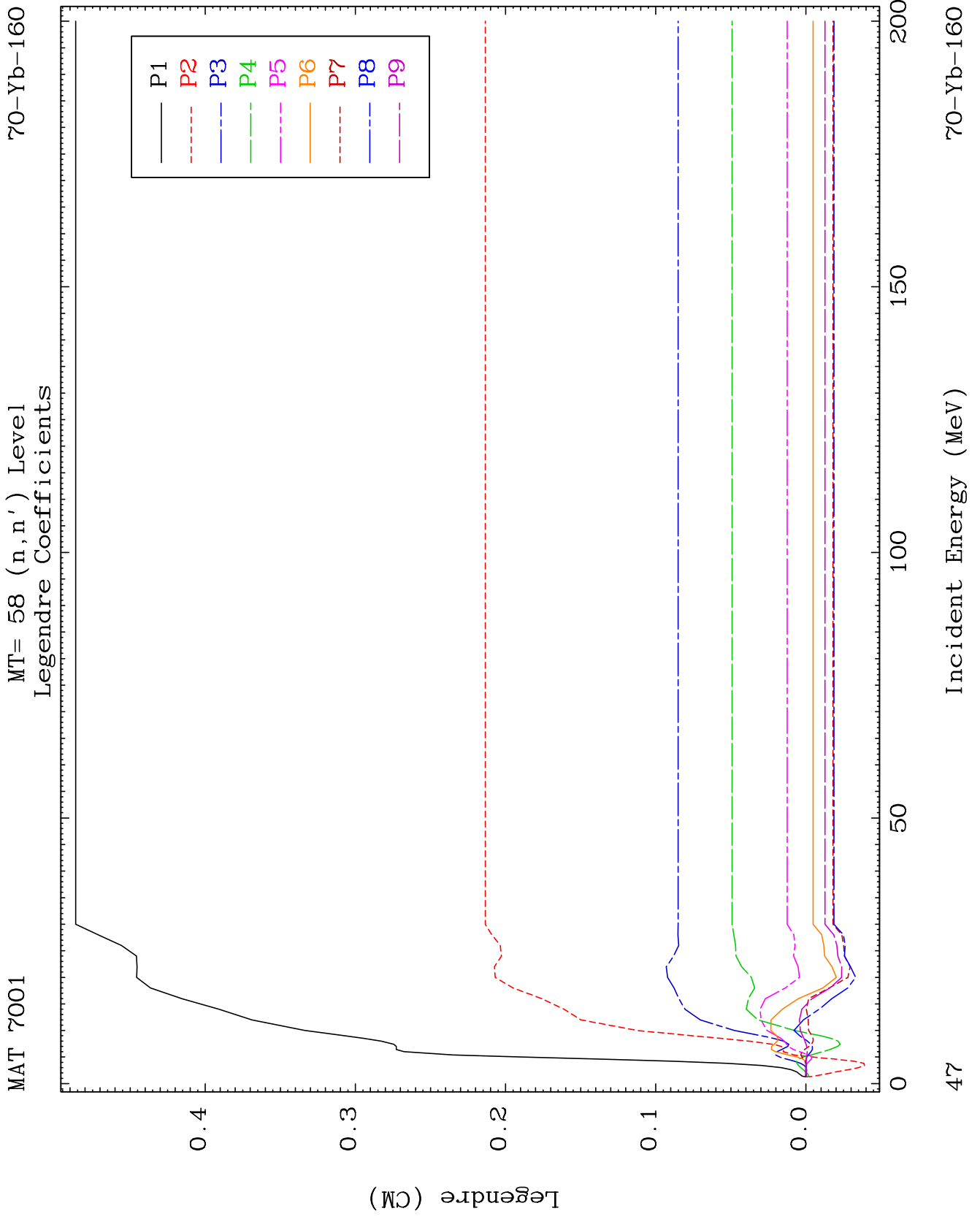


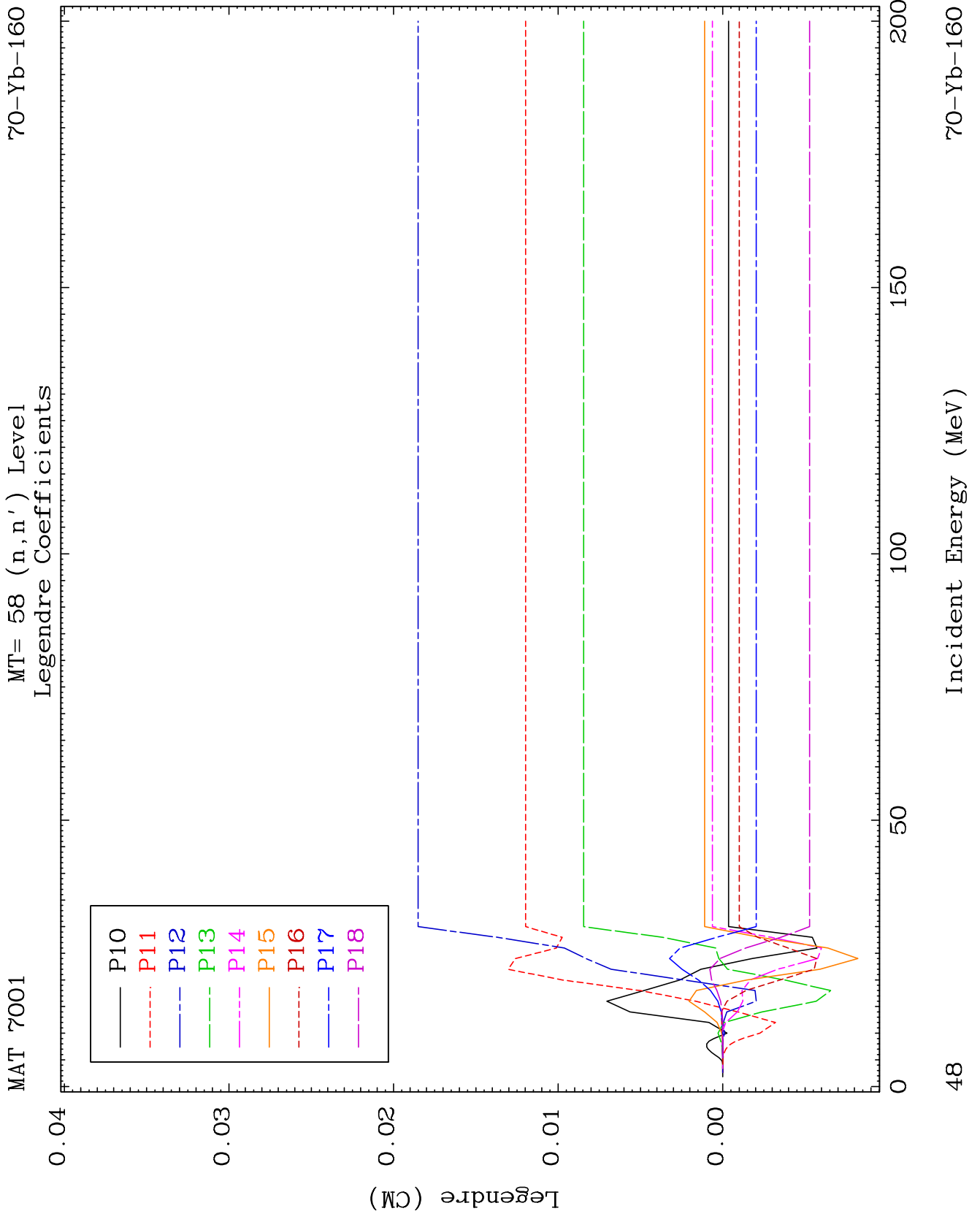








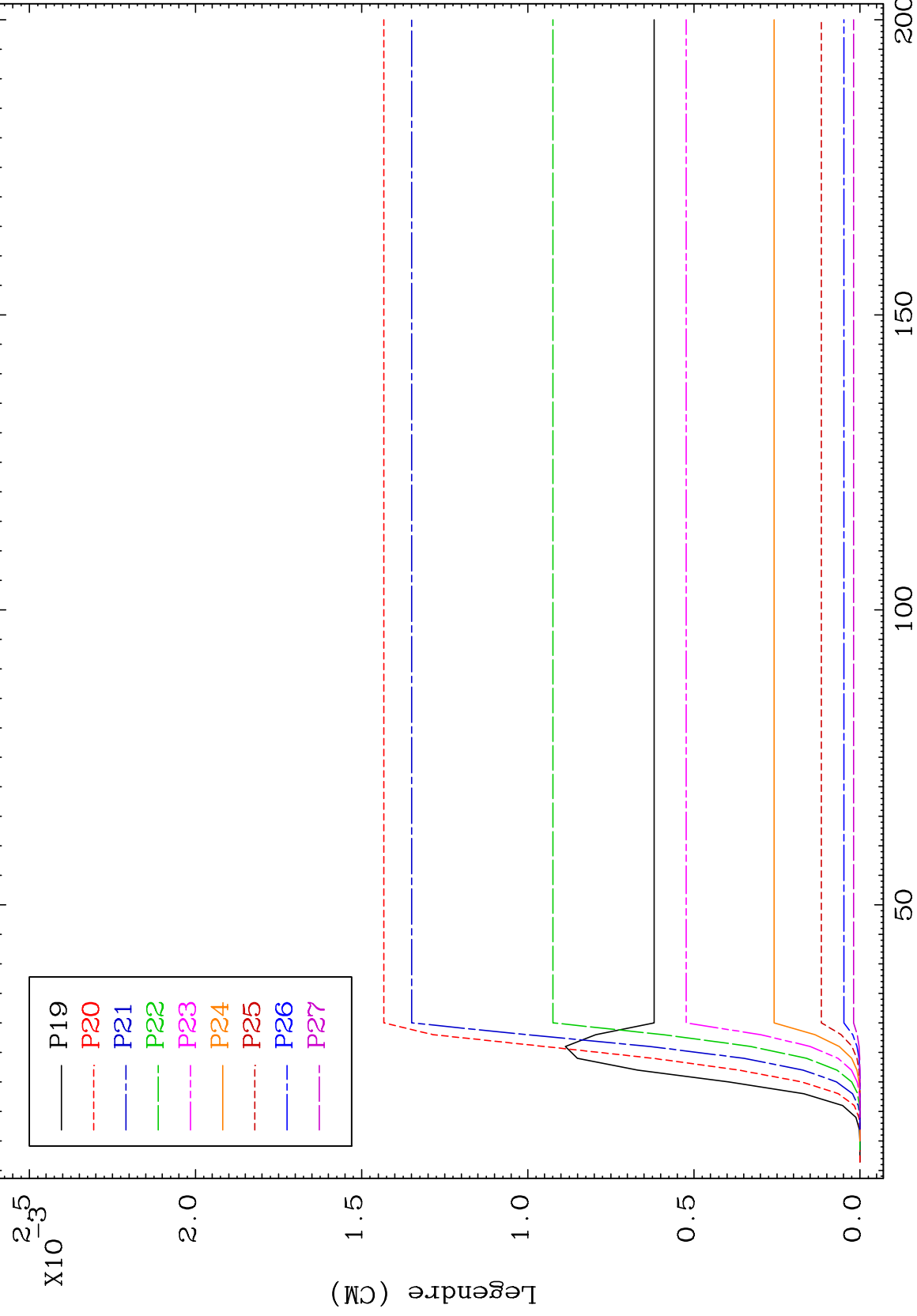




MAT 7001

MT= 58 (n,n') Level
Legendre Coefficients

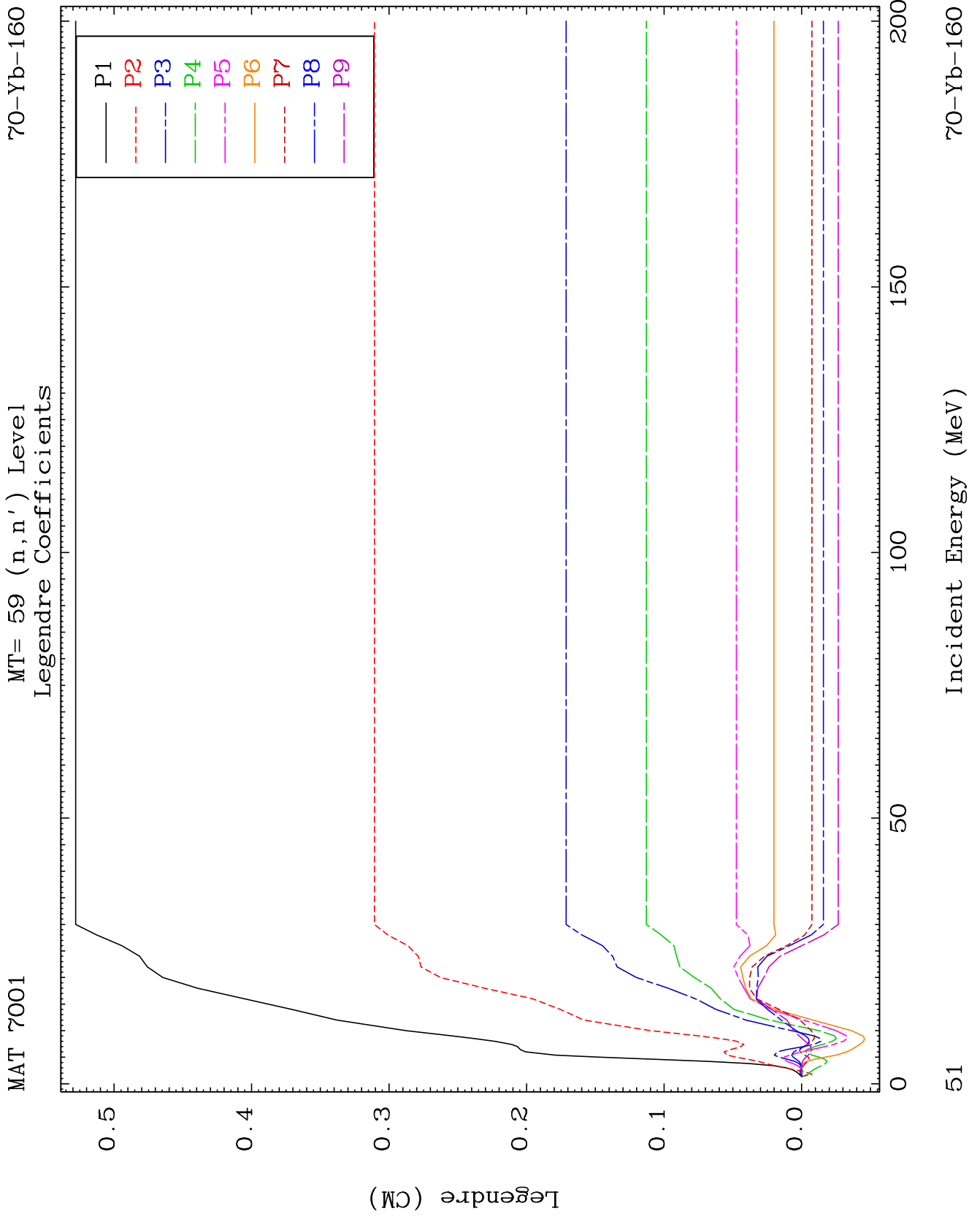
70-Yb-160

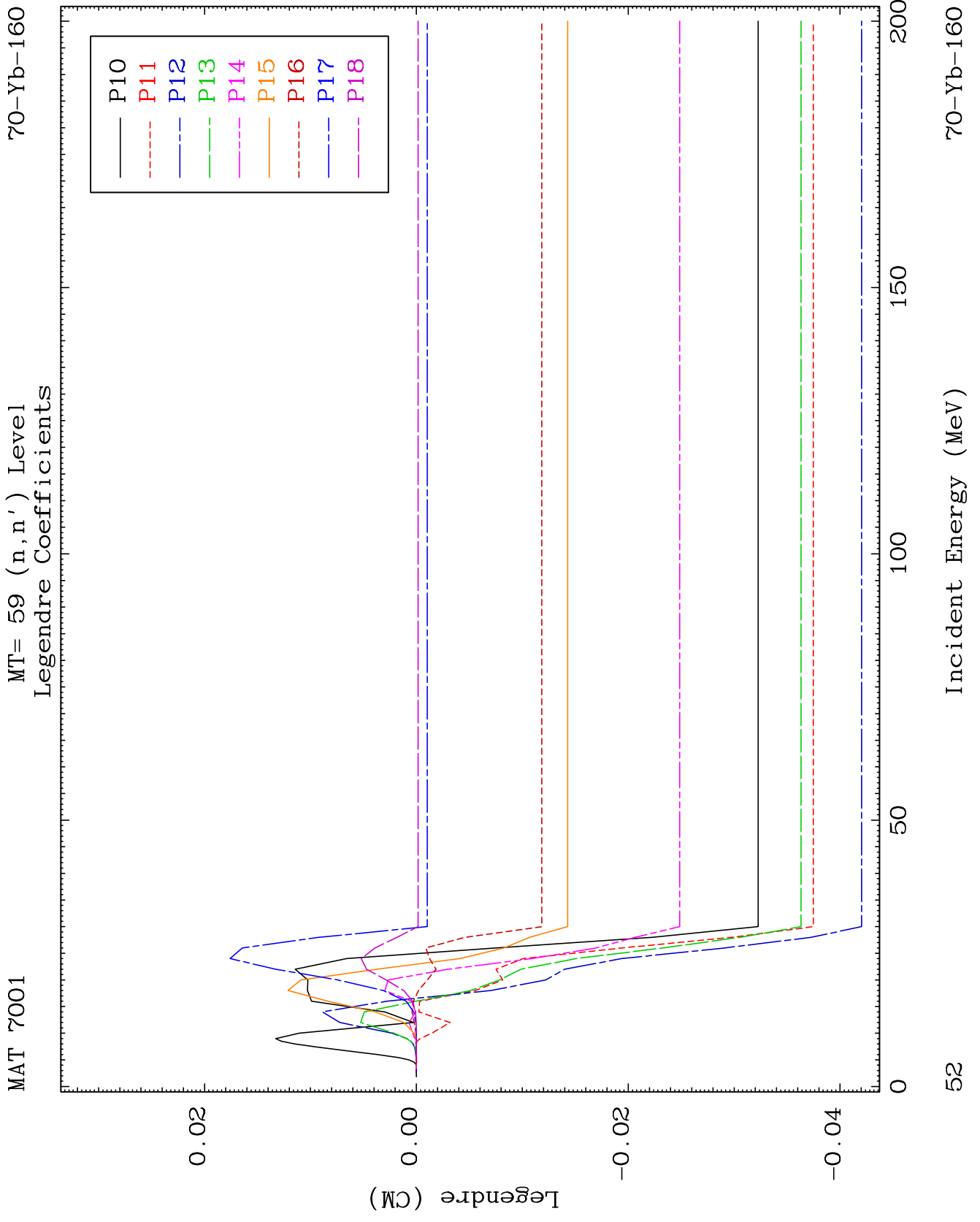


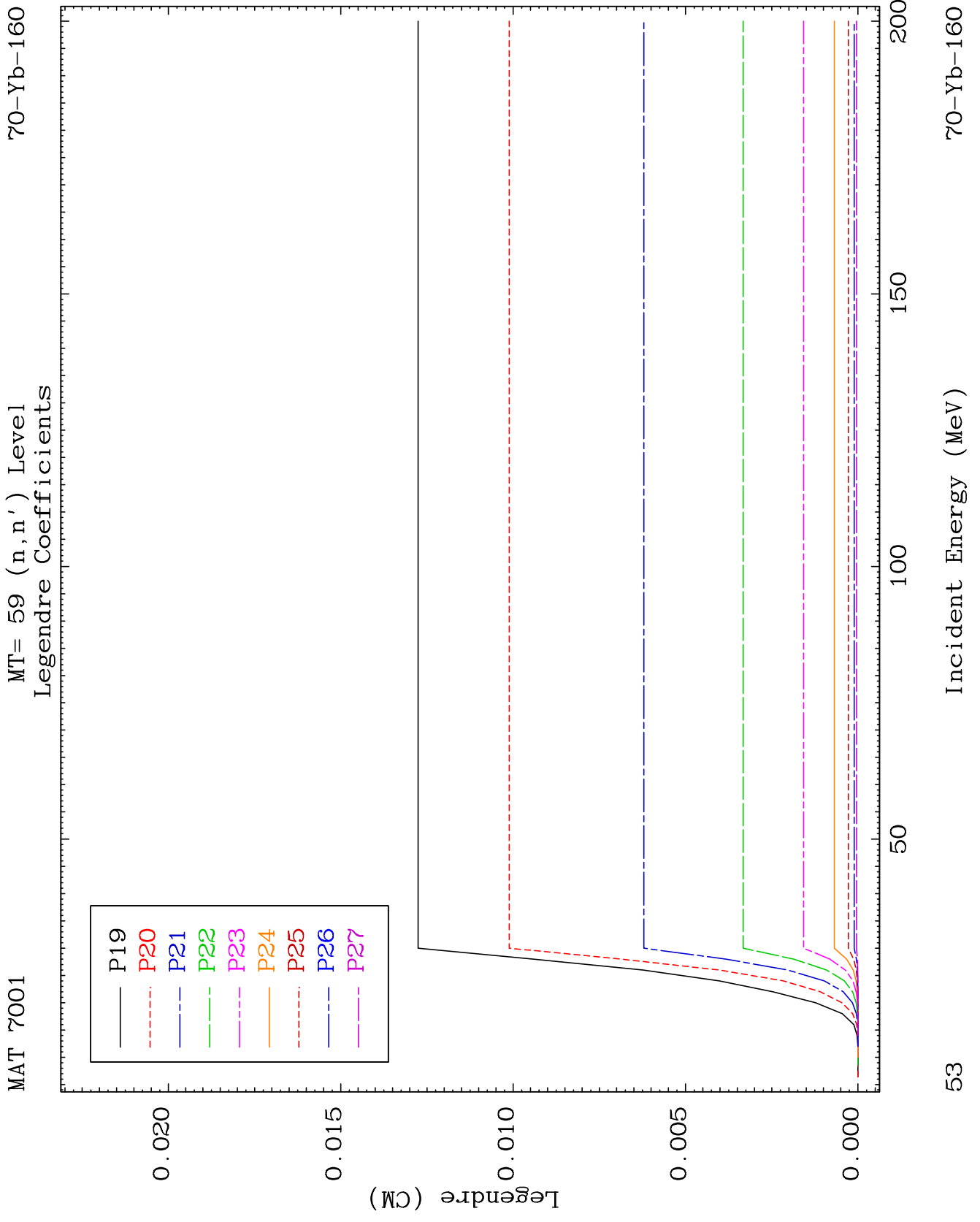
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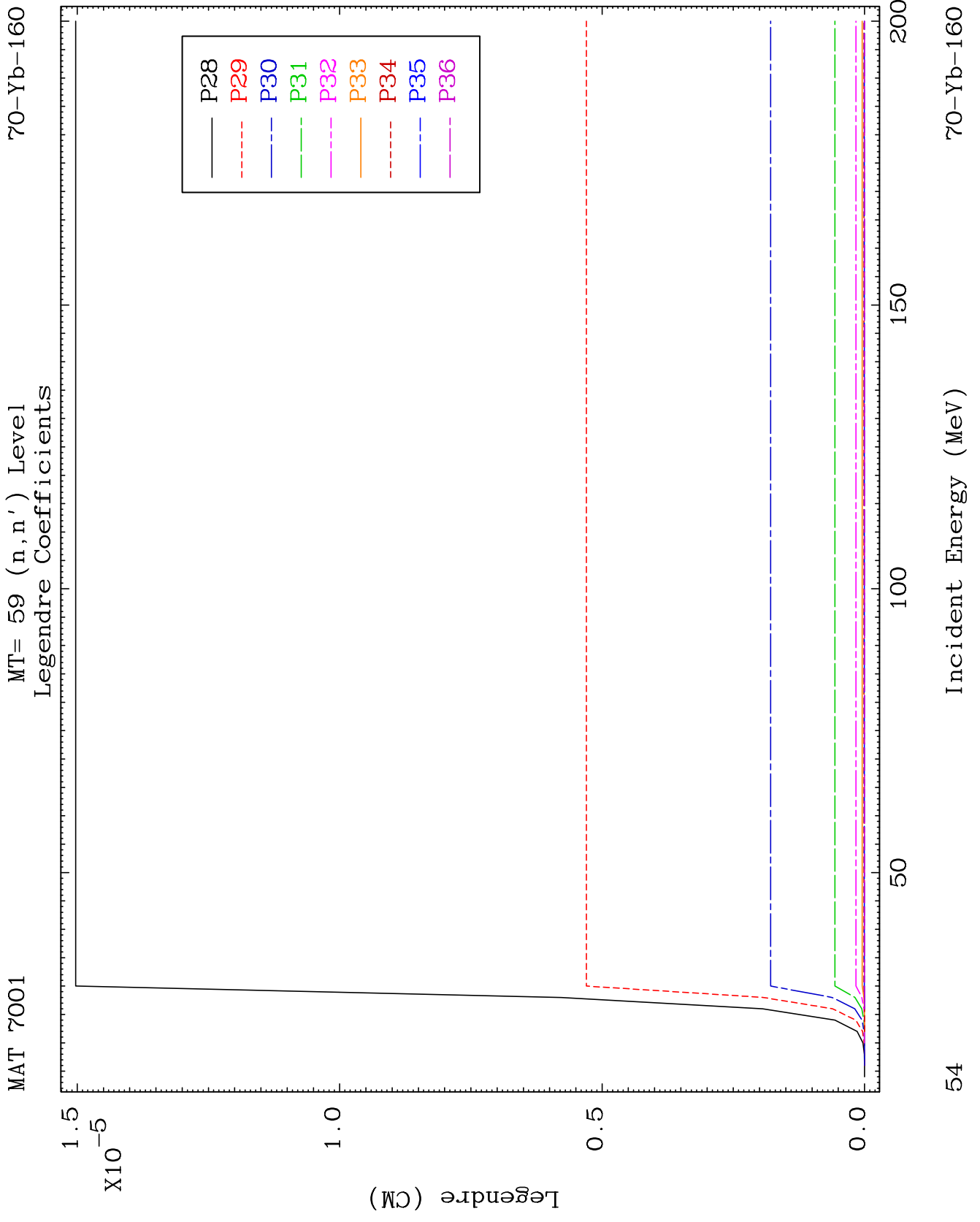
Incident Energy (MeV)

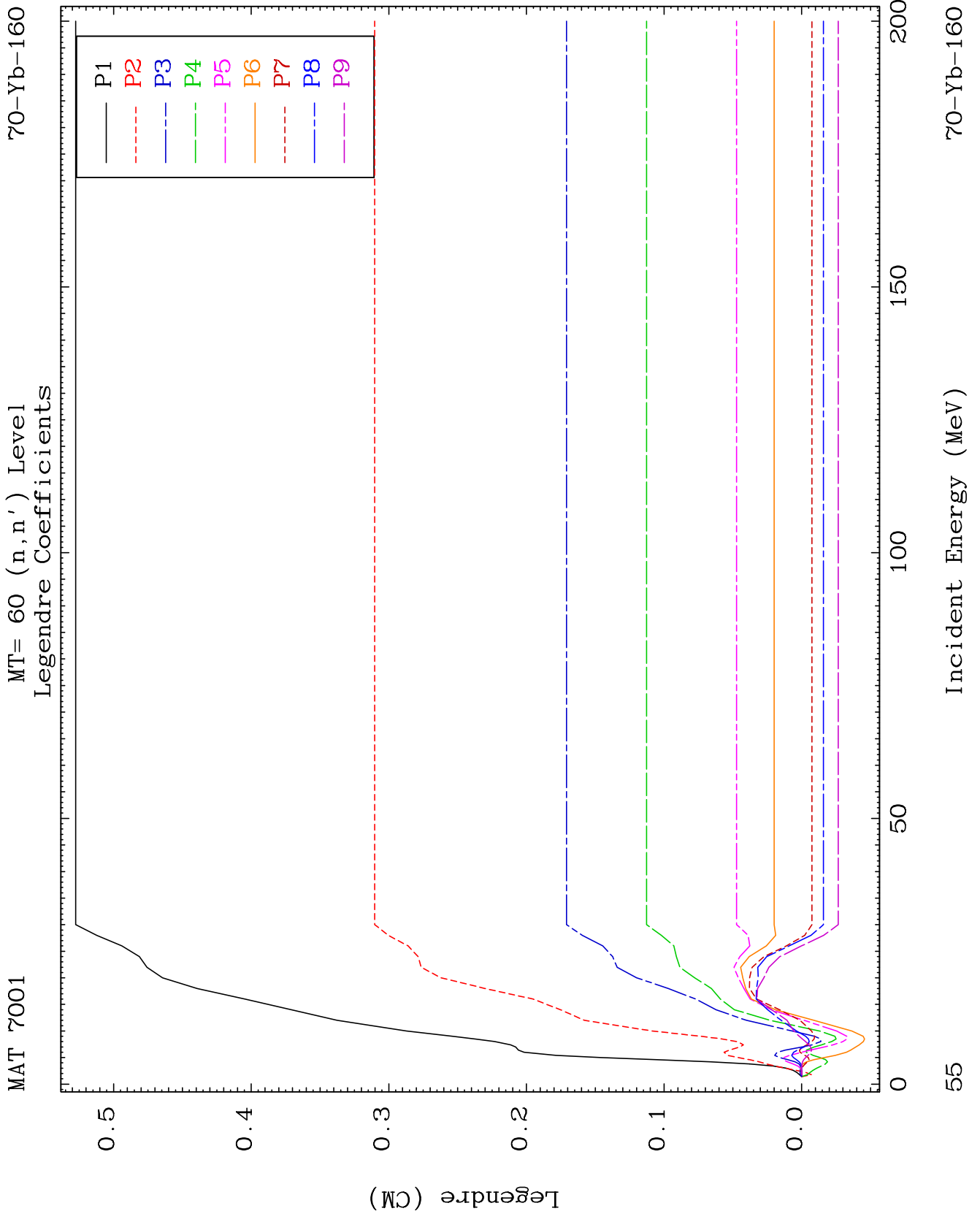
70-Yb-160

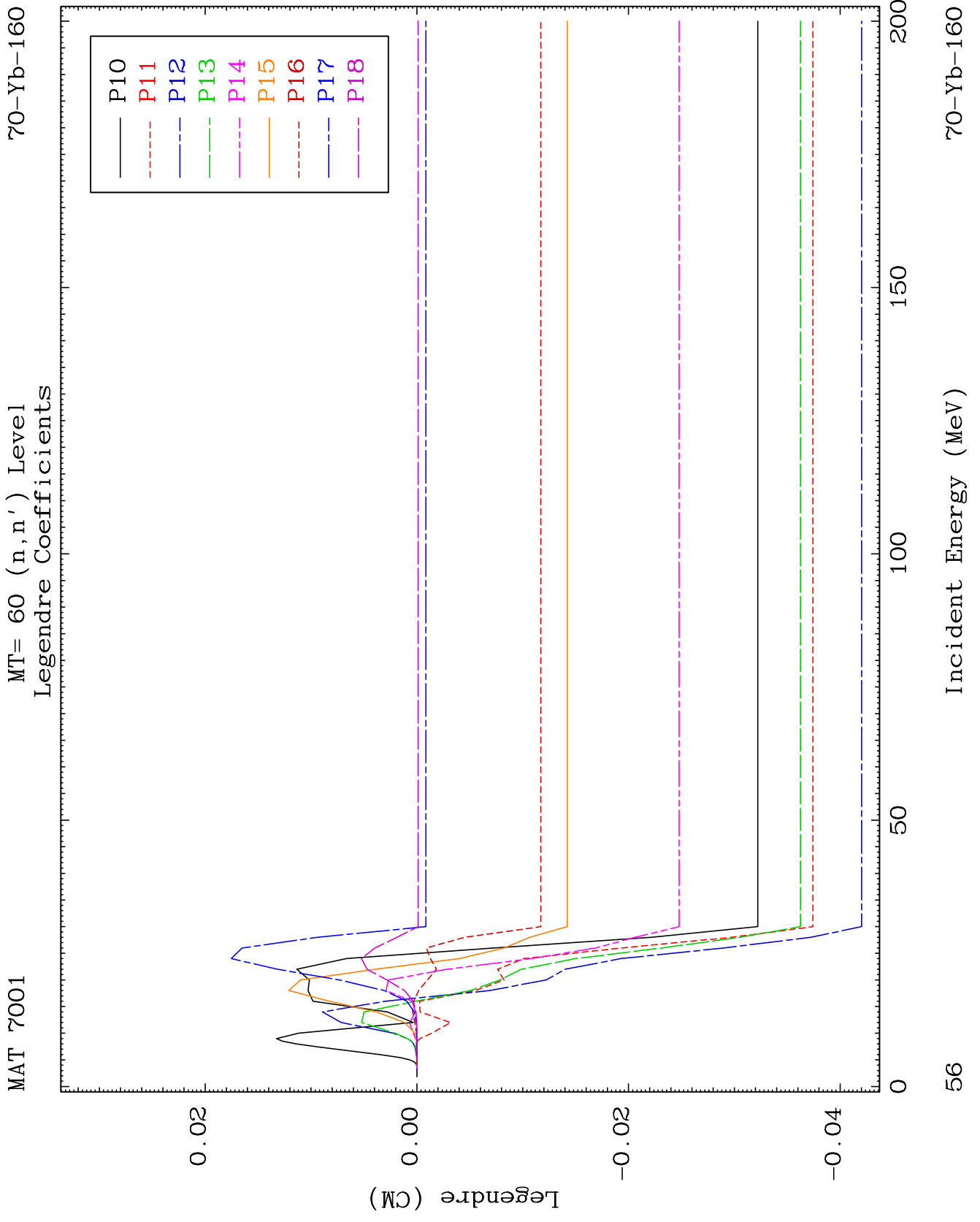


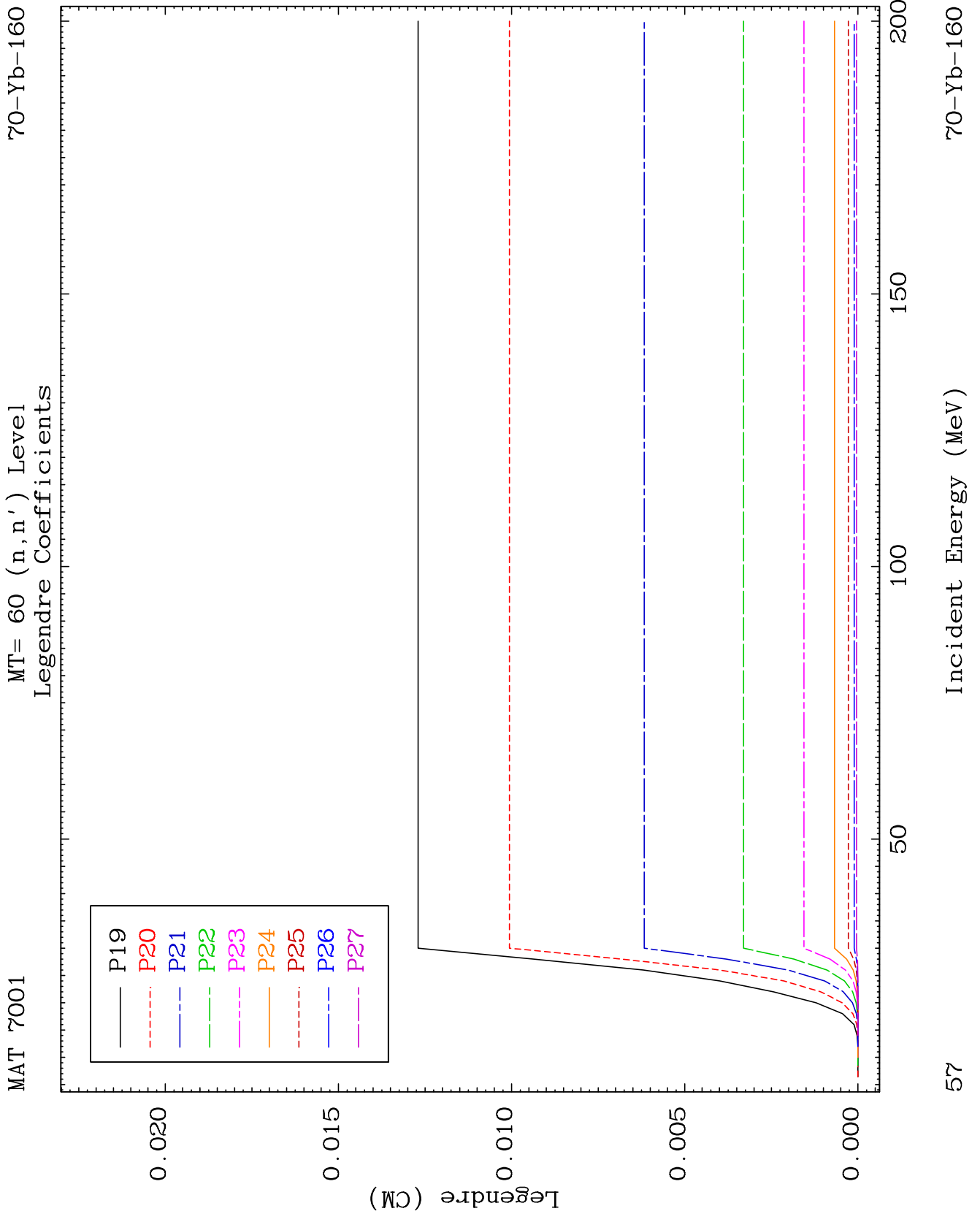


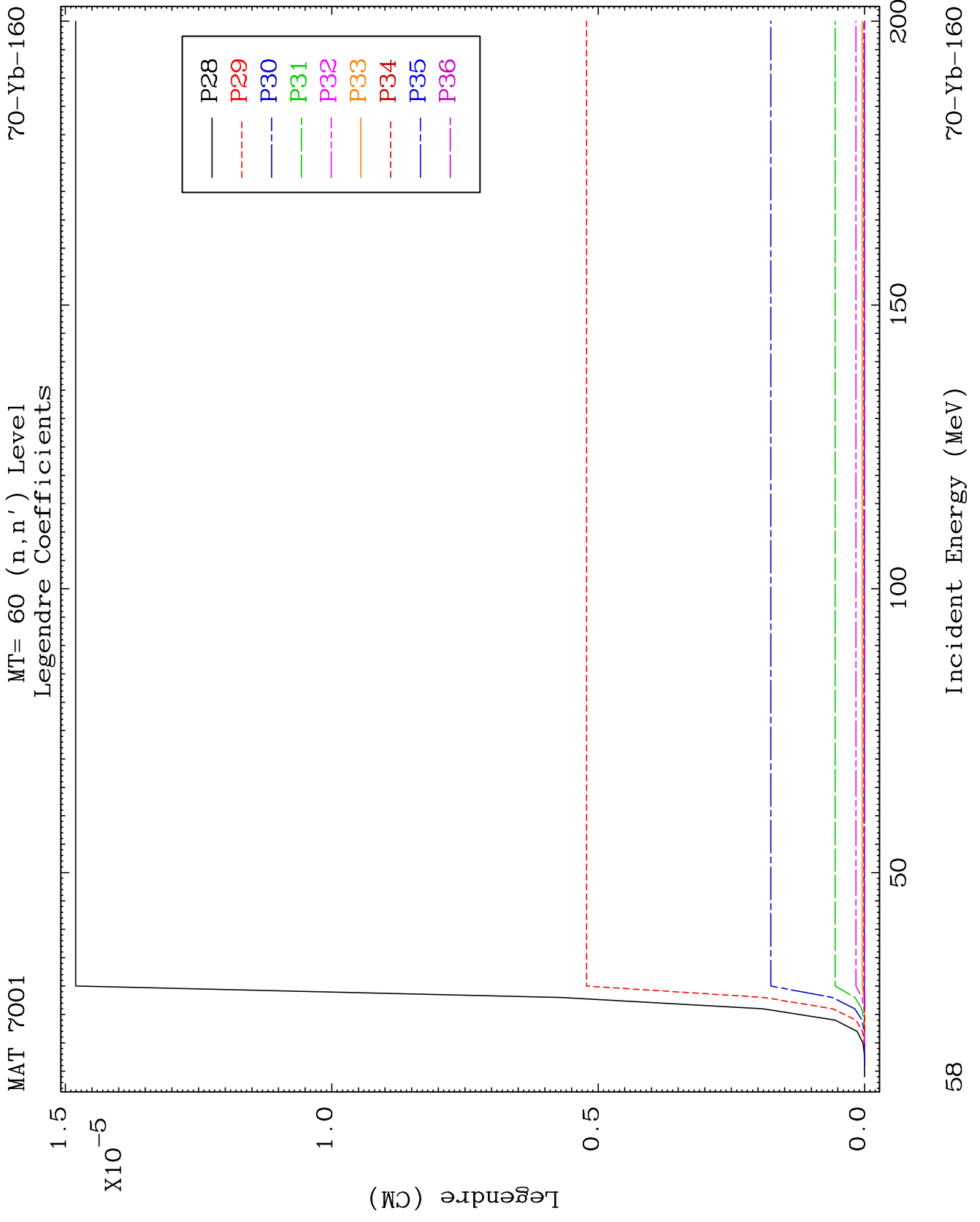


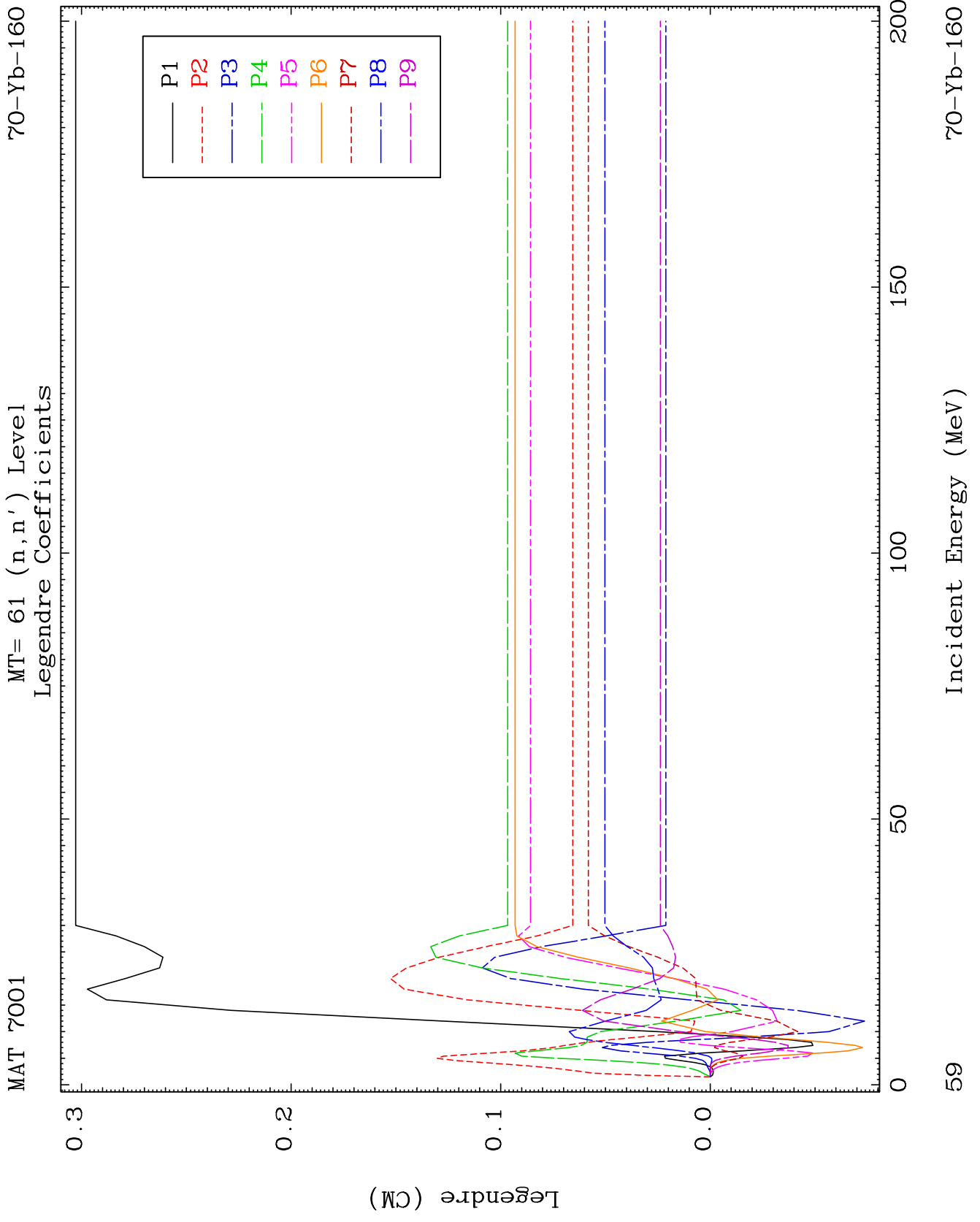


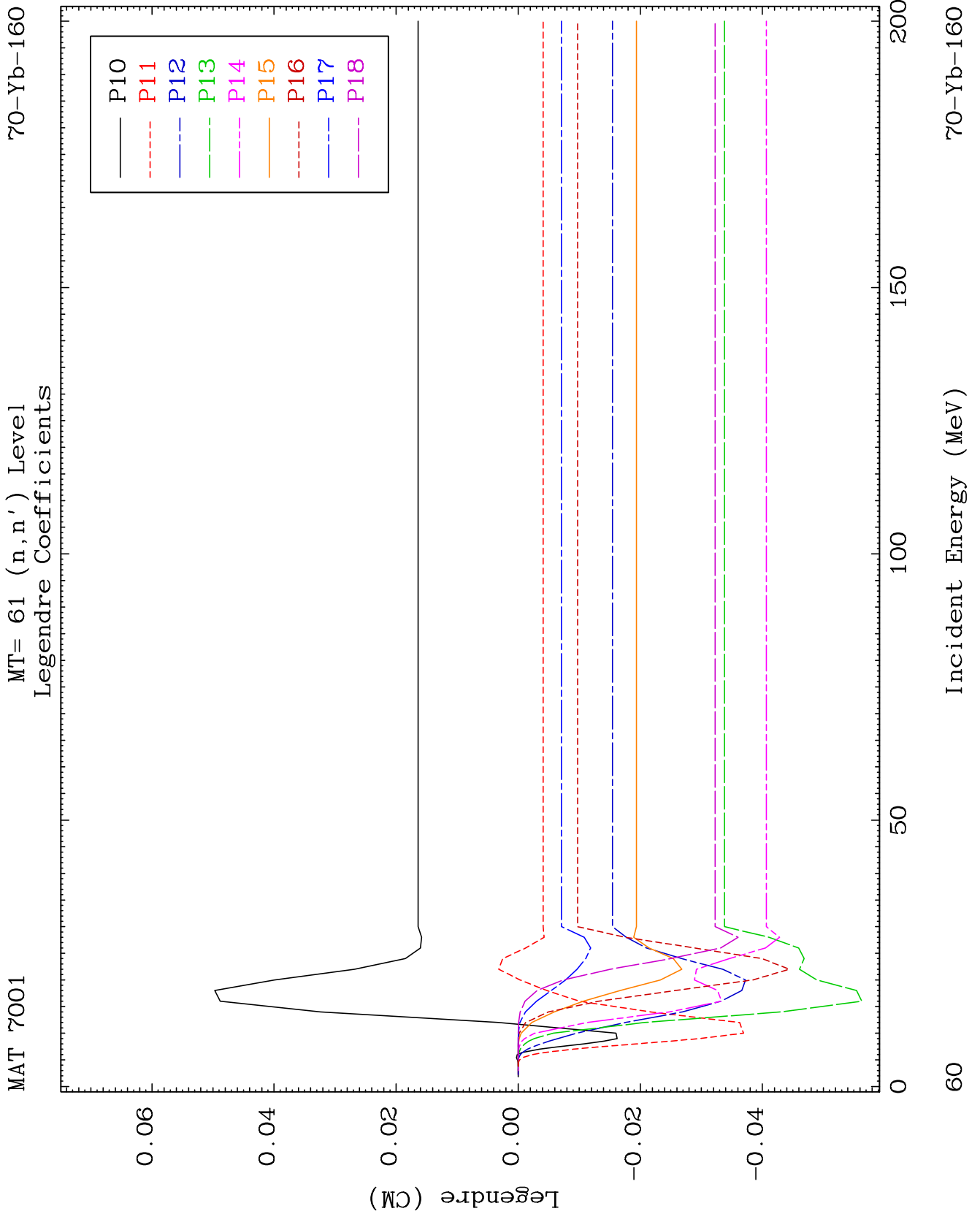


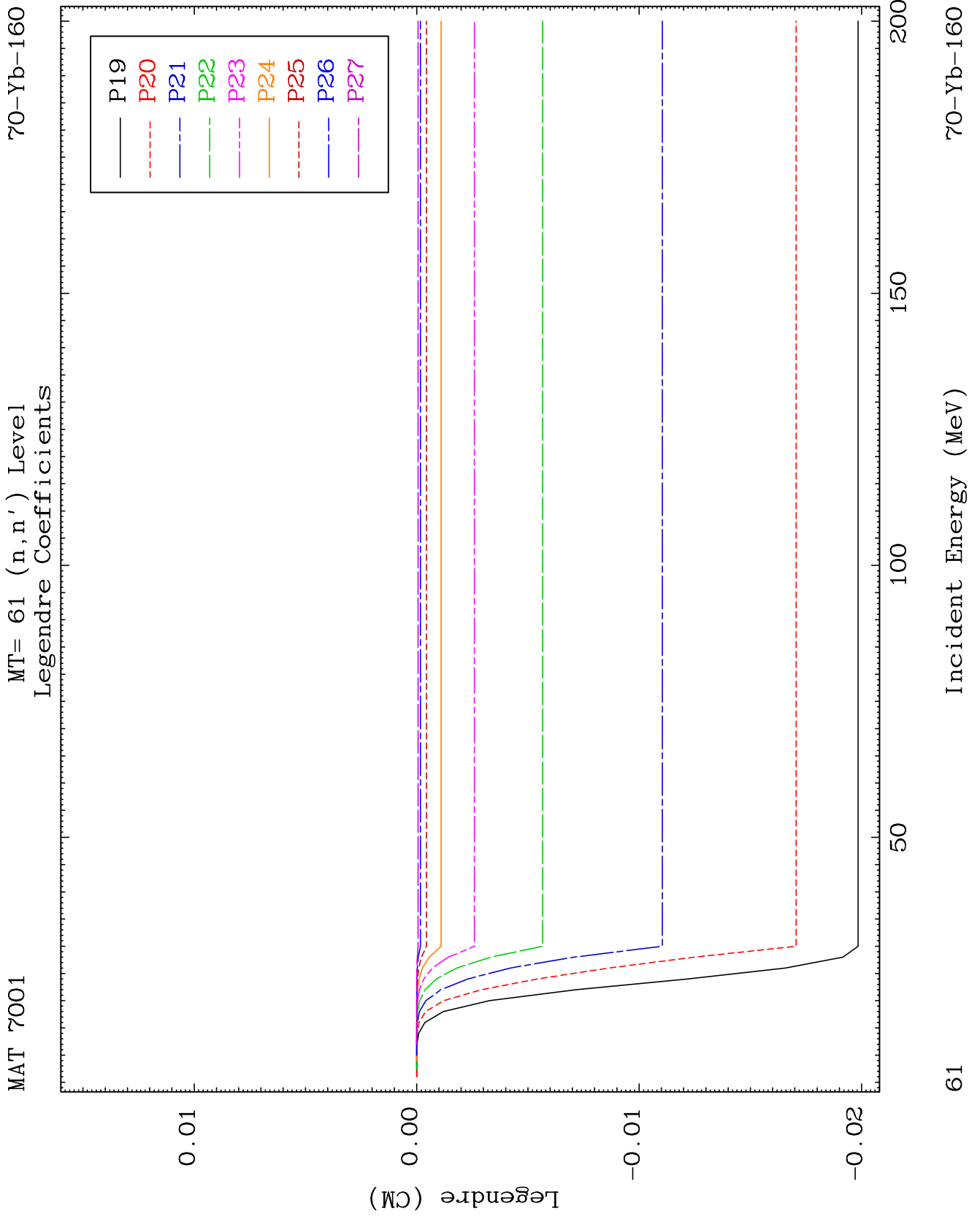


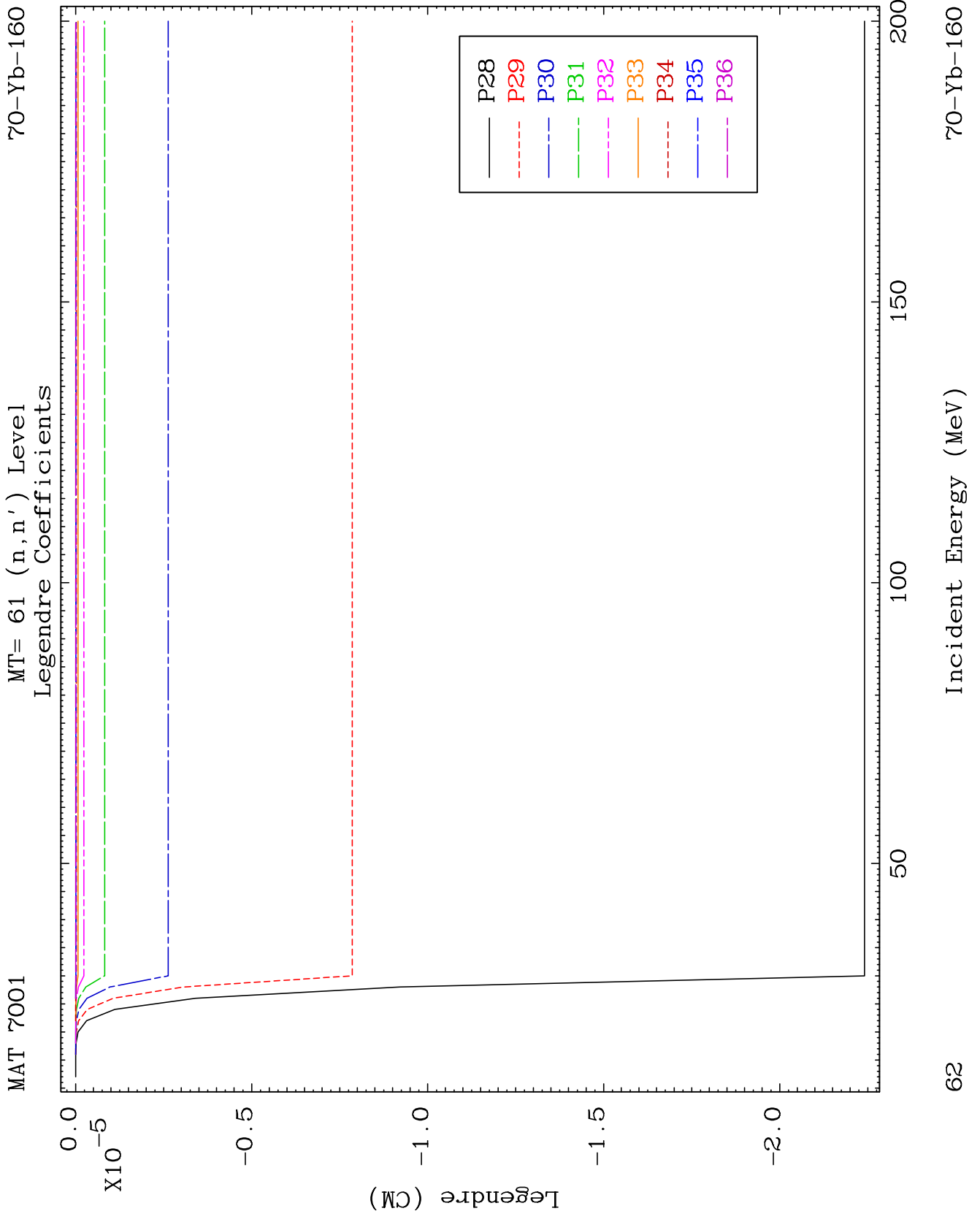


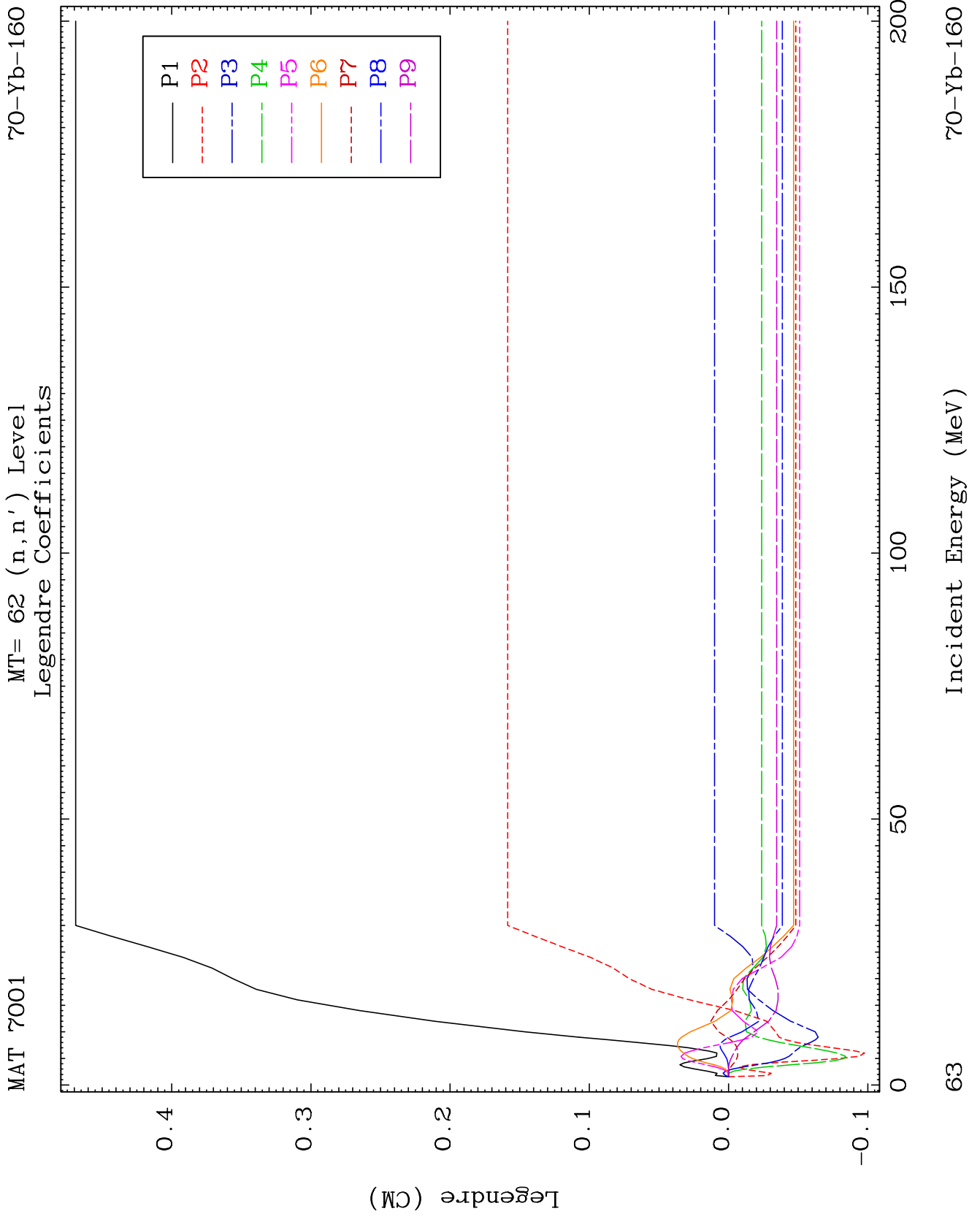


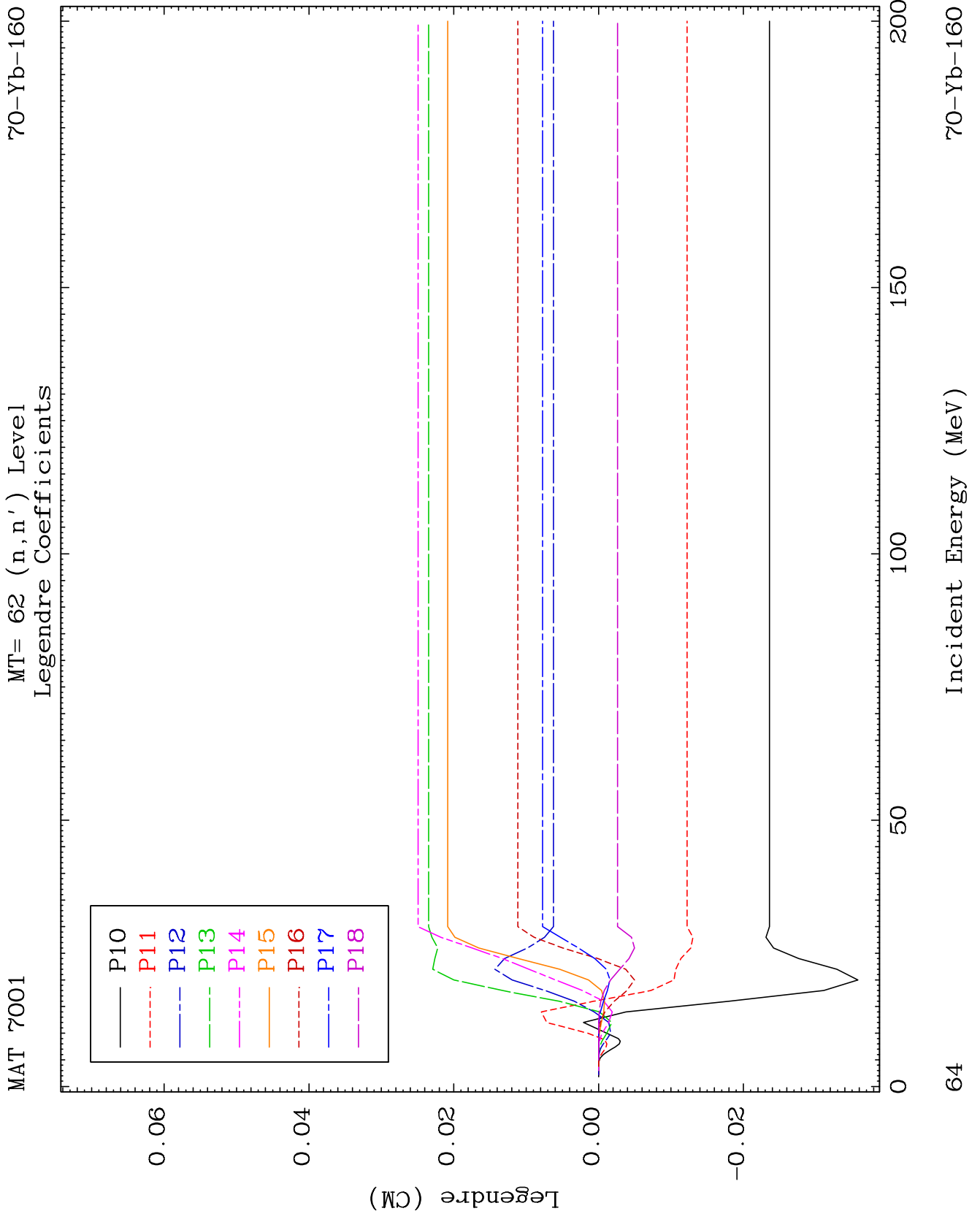


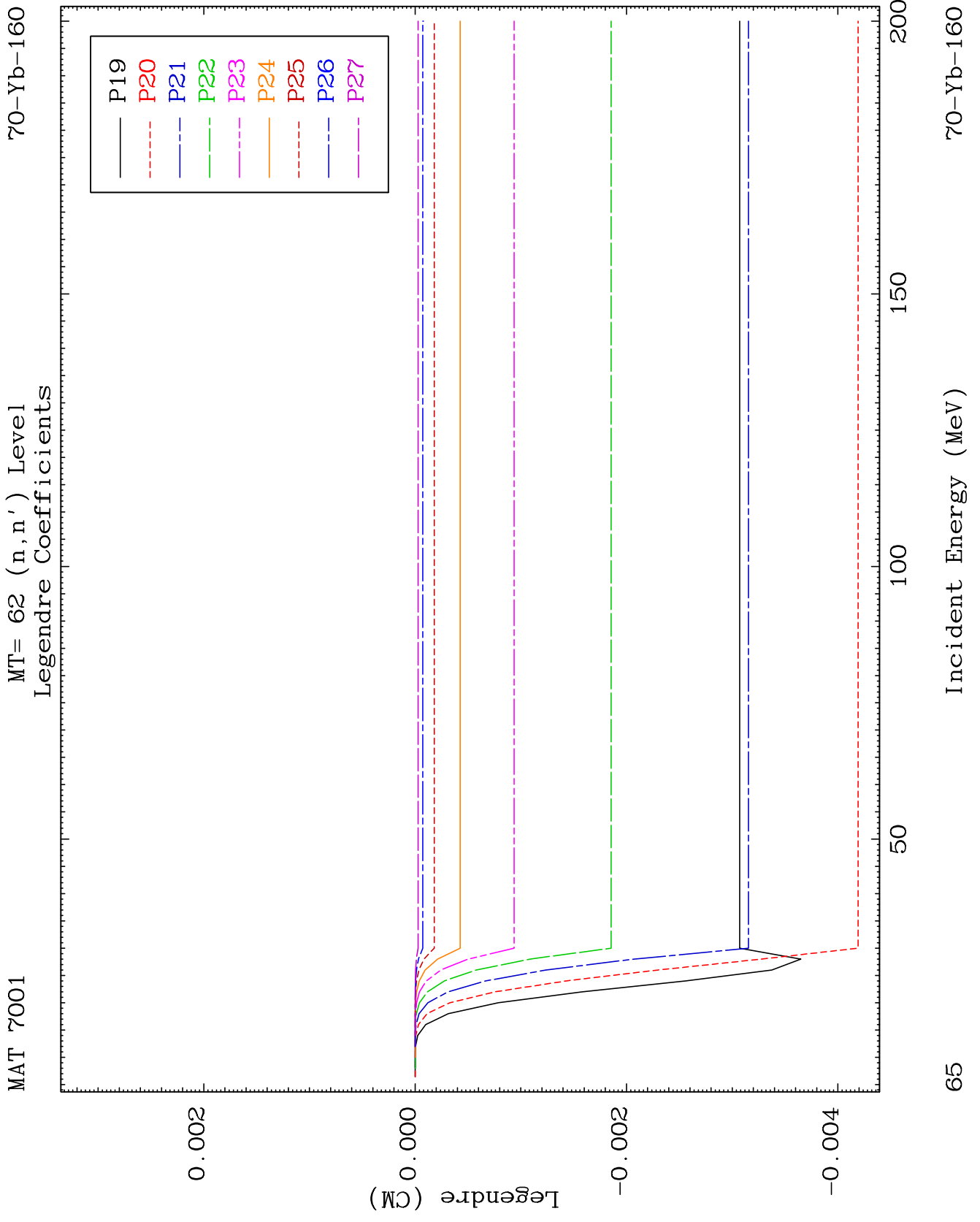


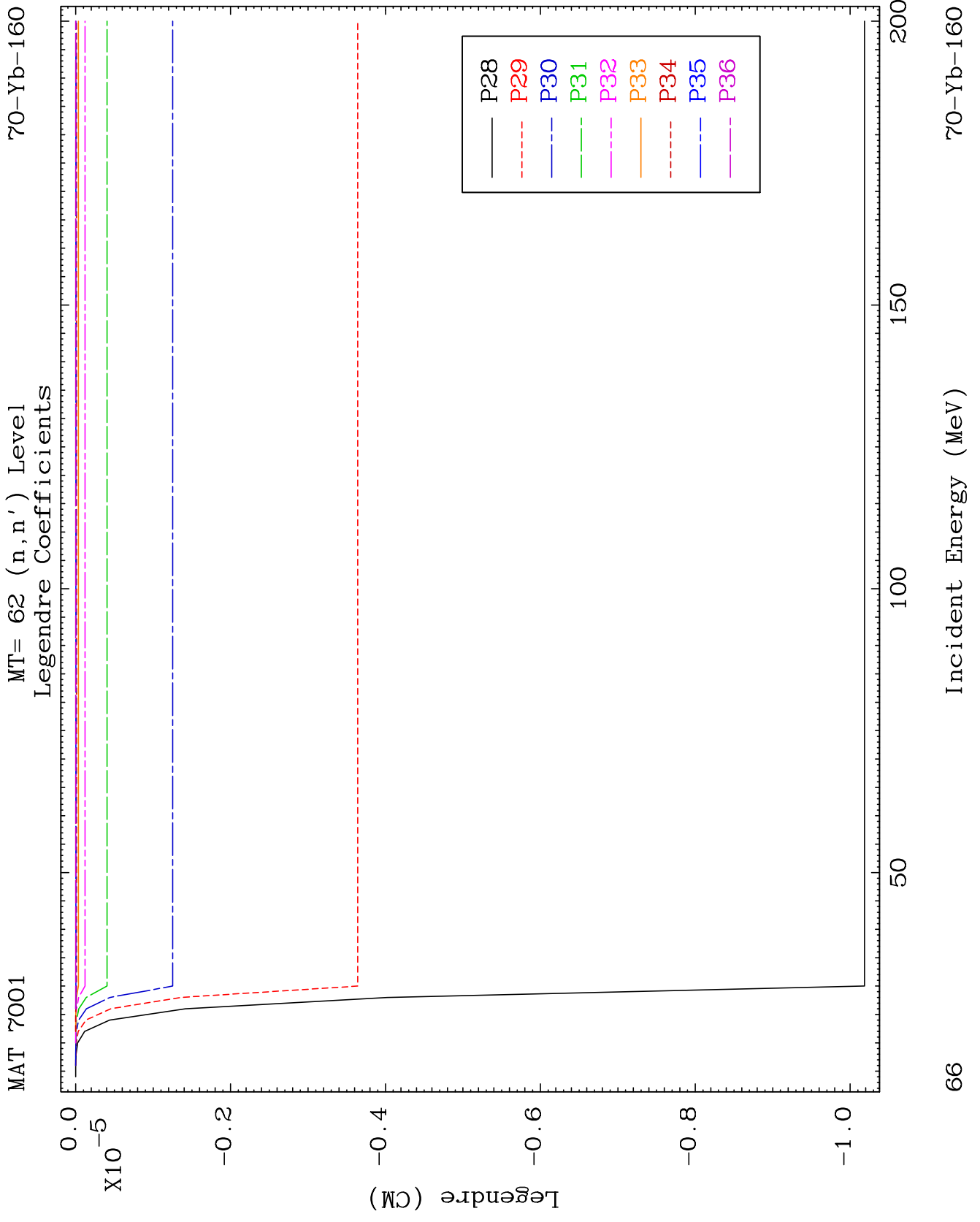


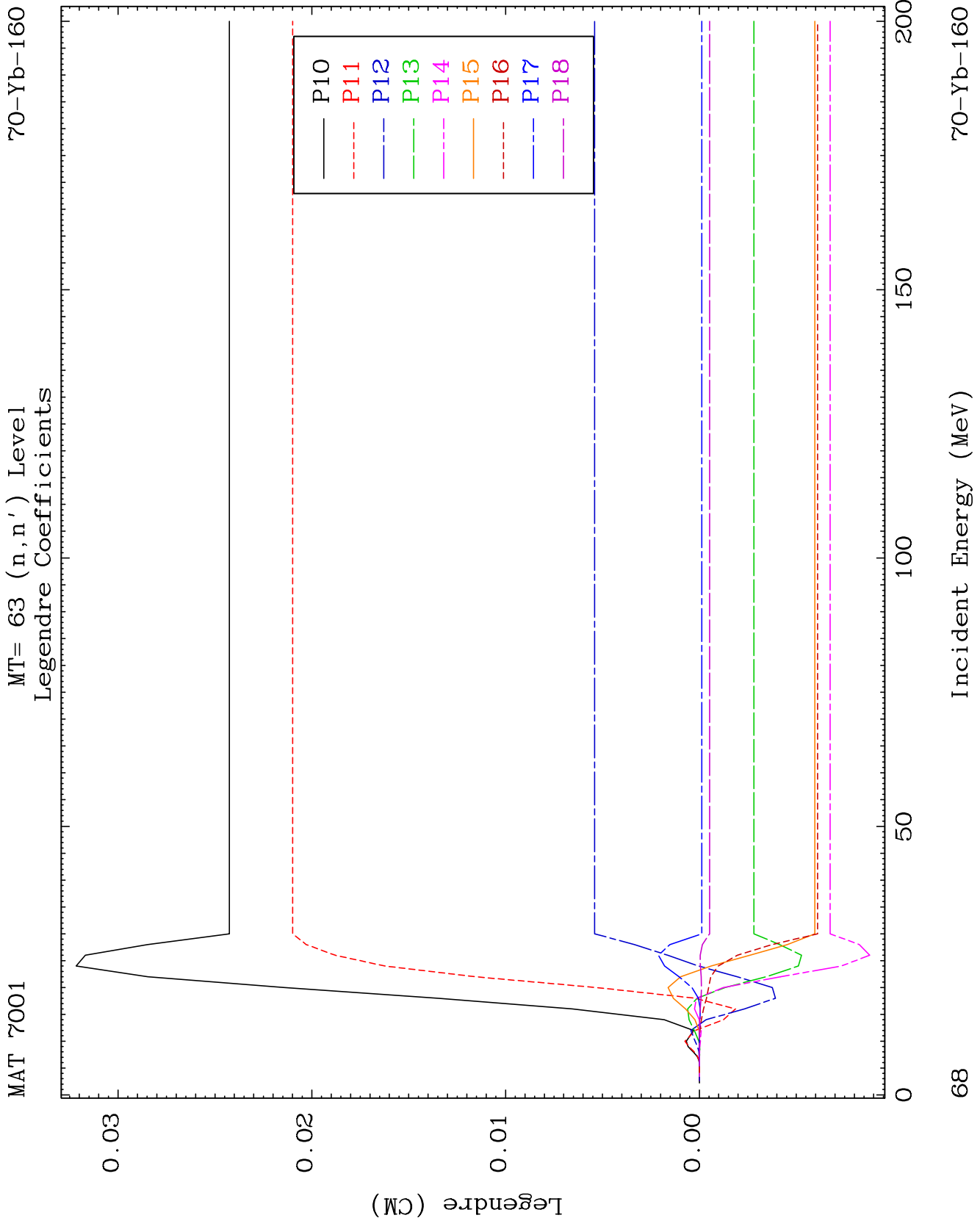


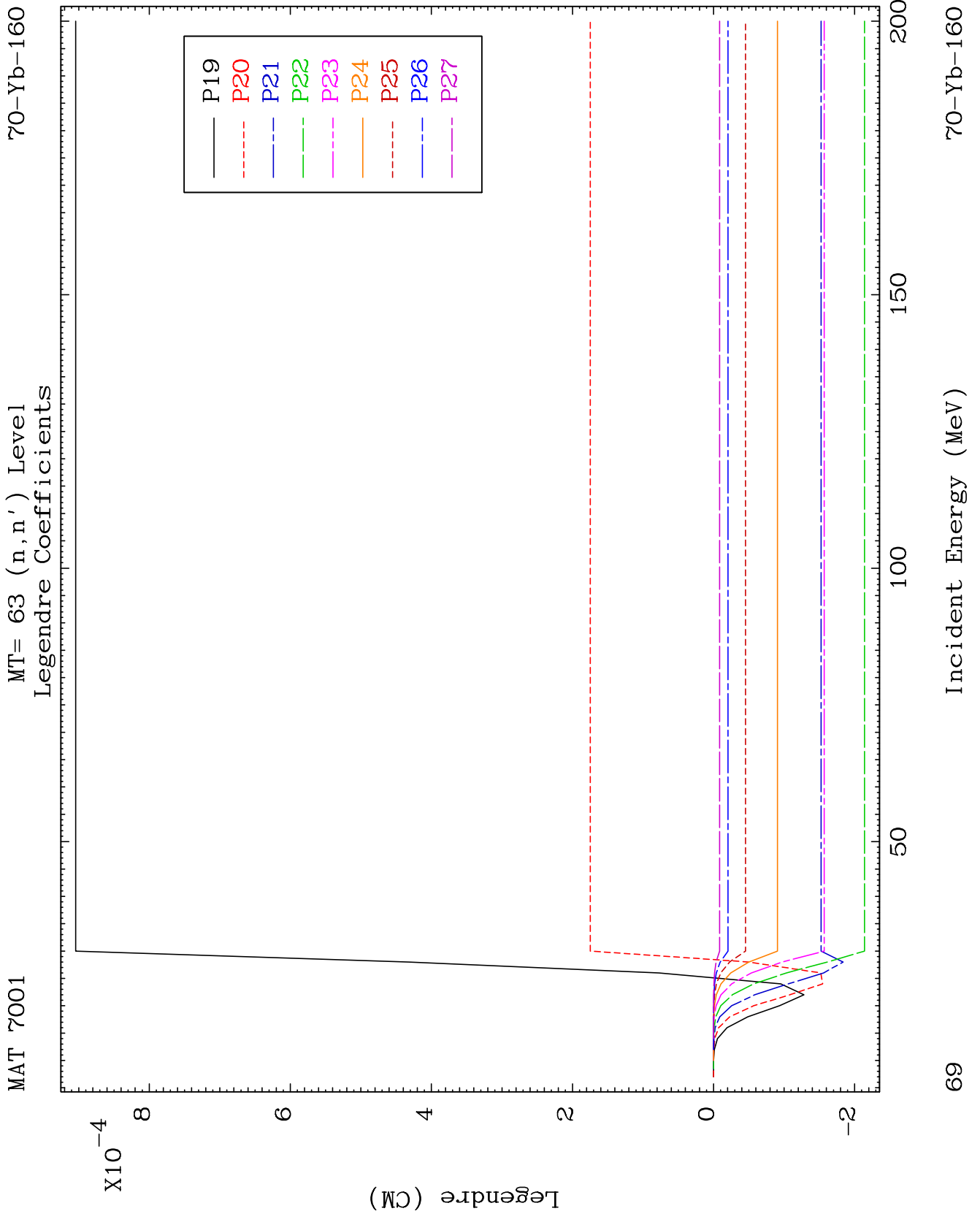


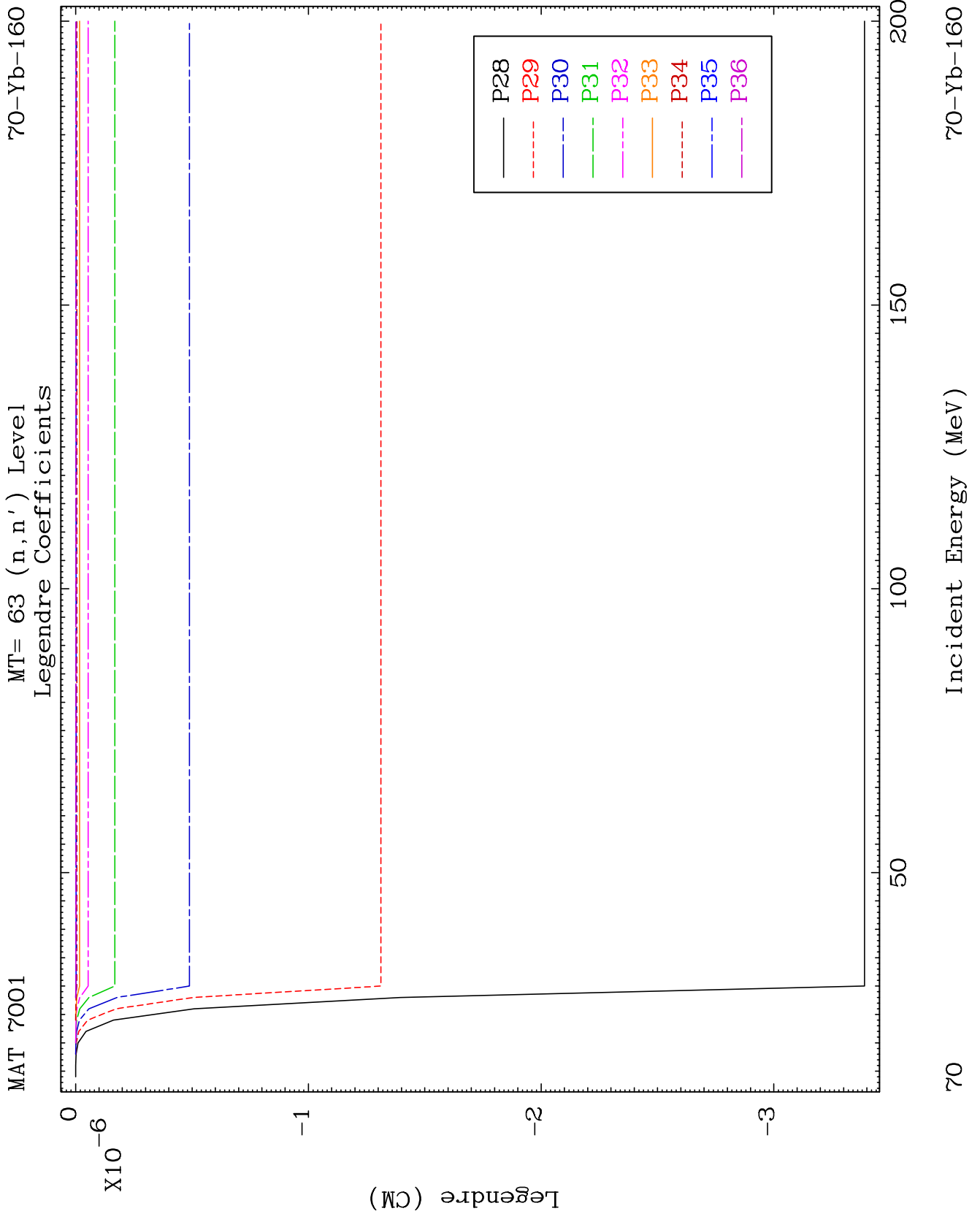


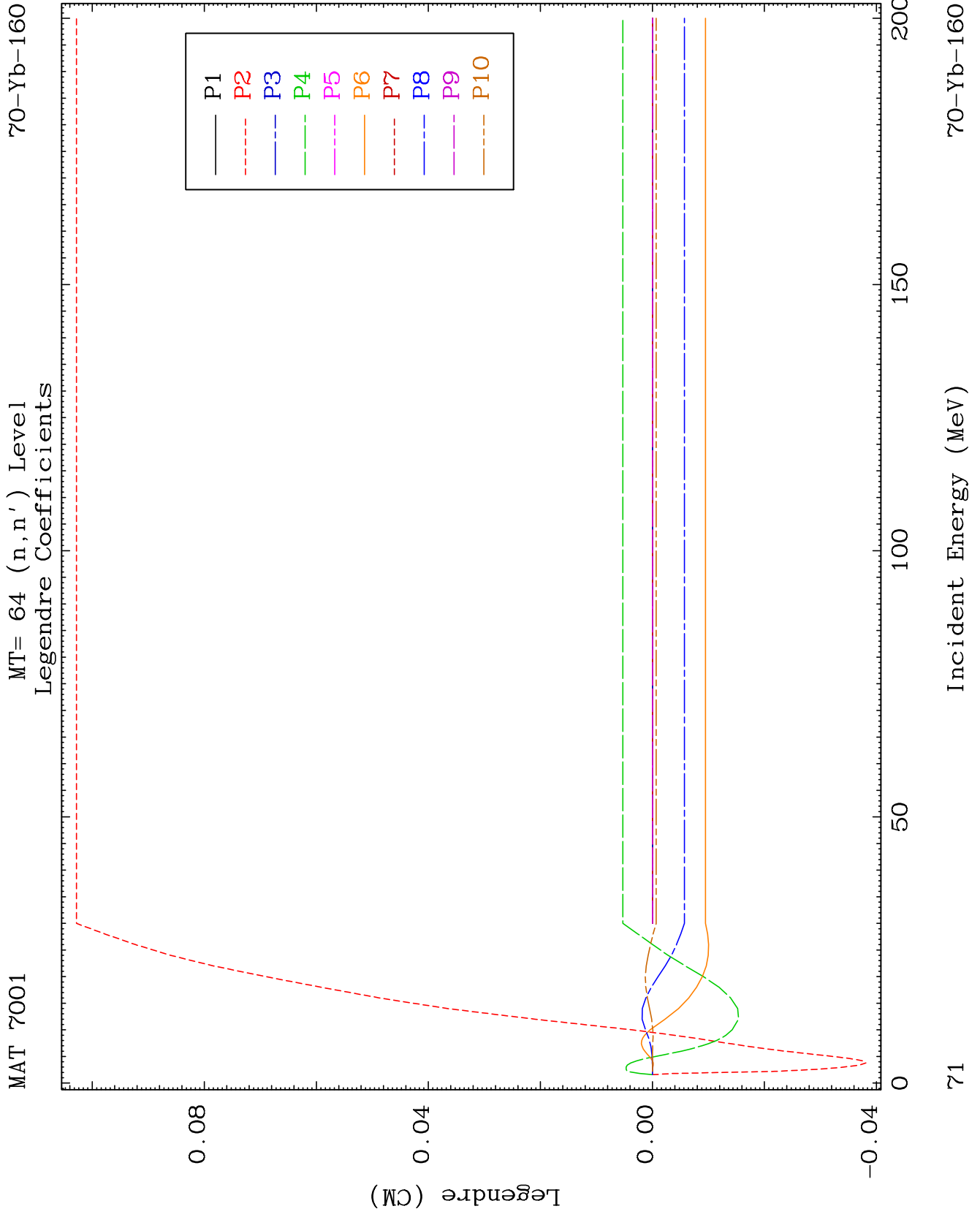


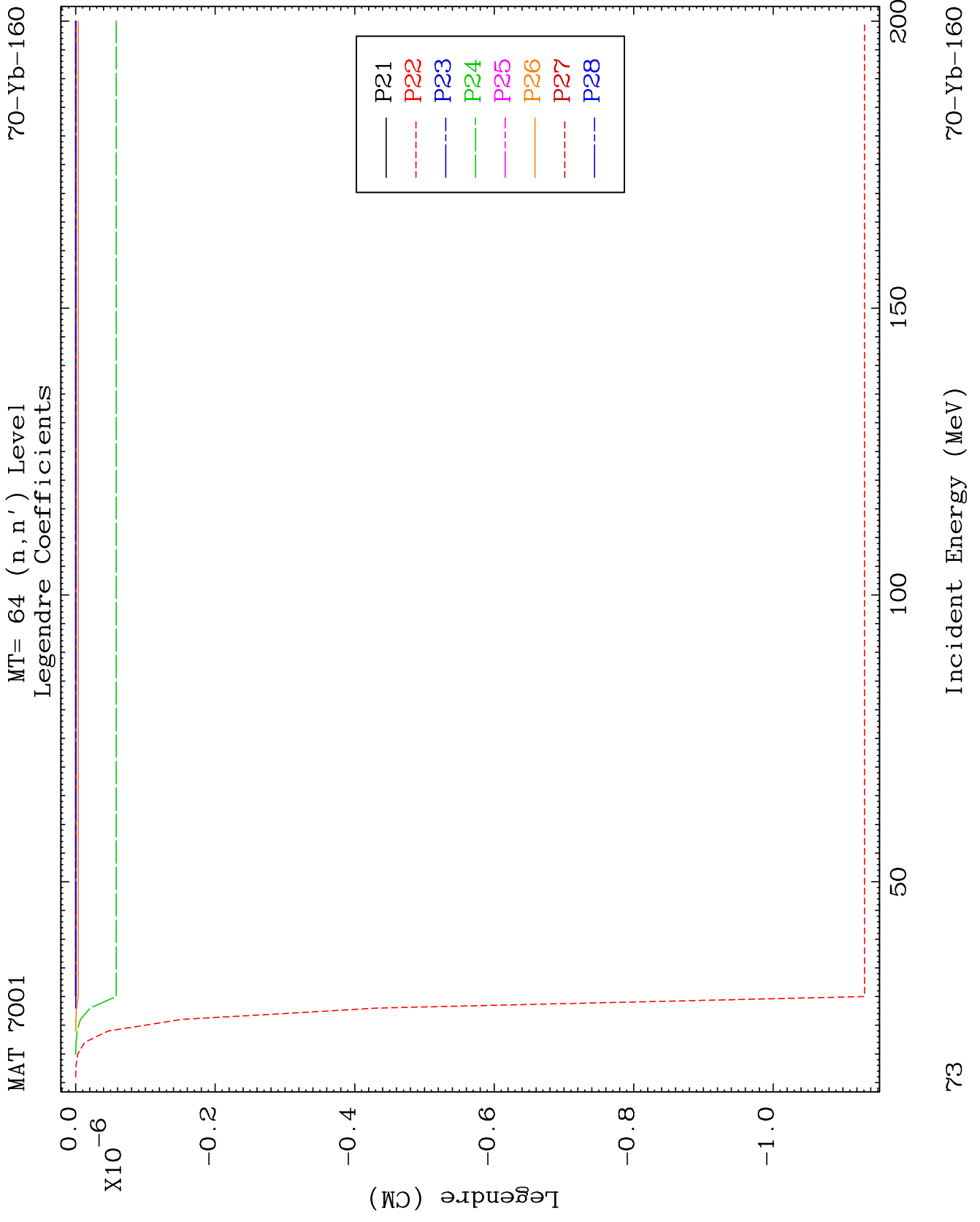


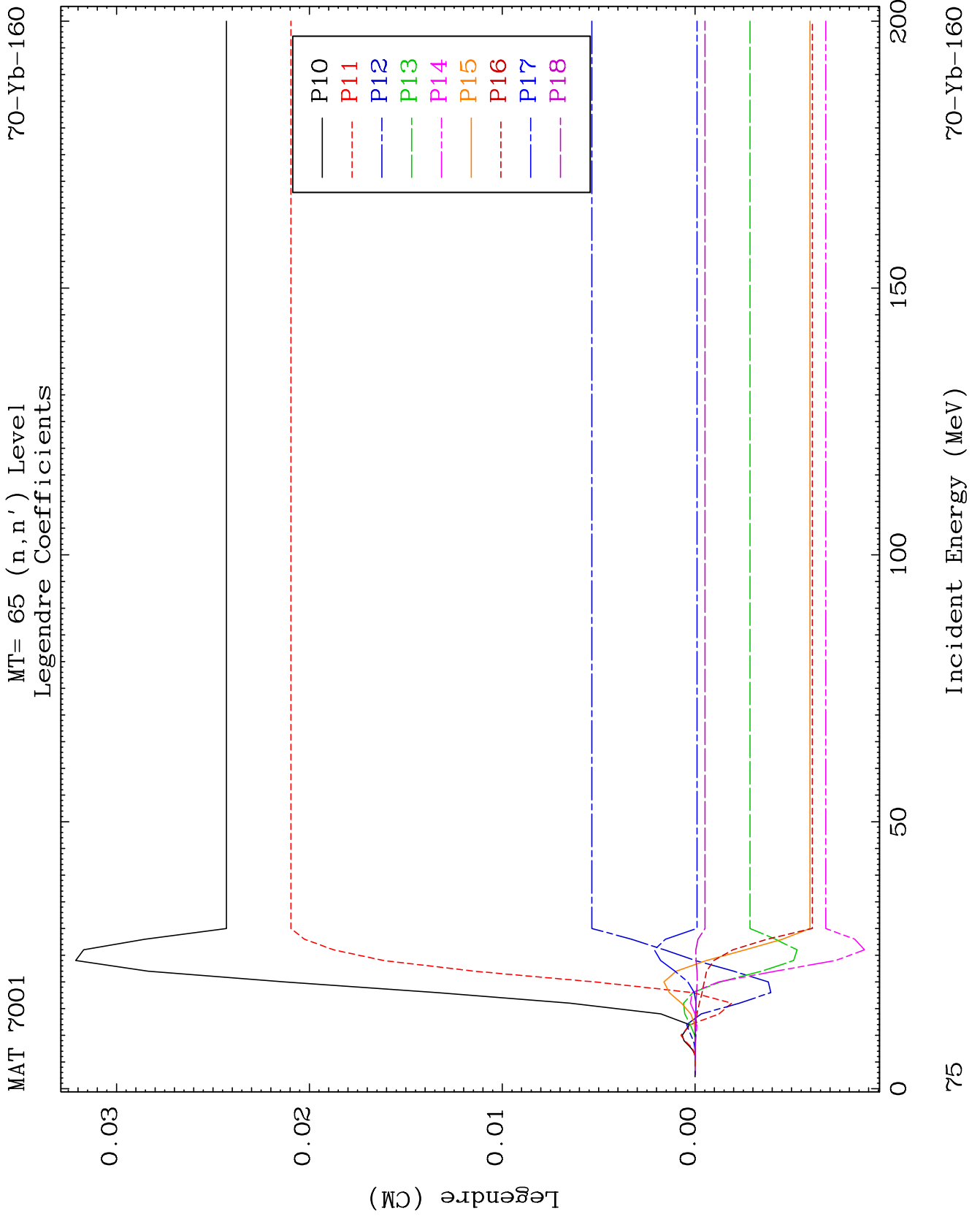


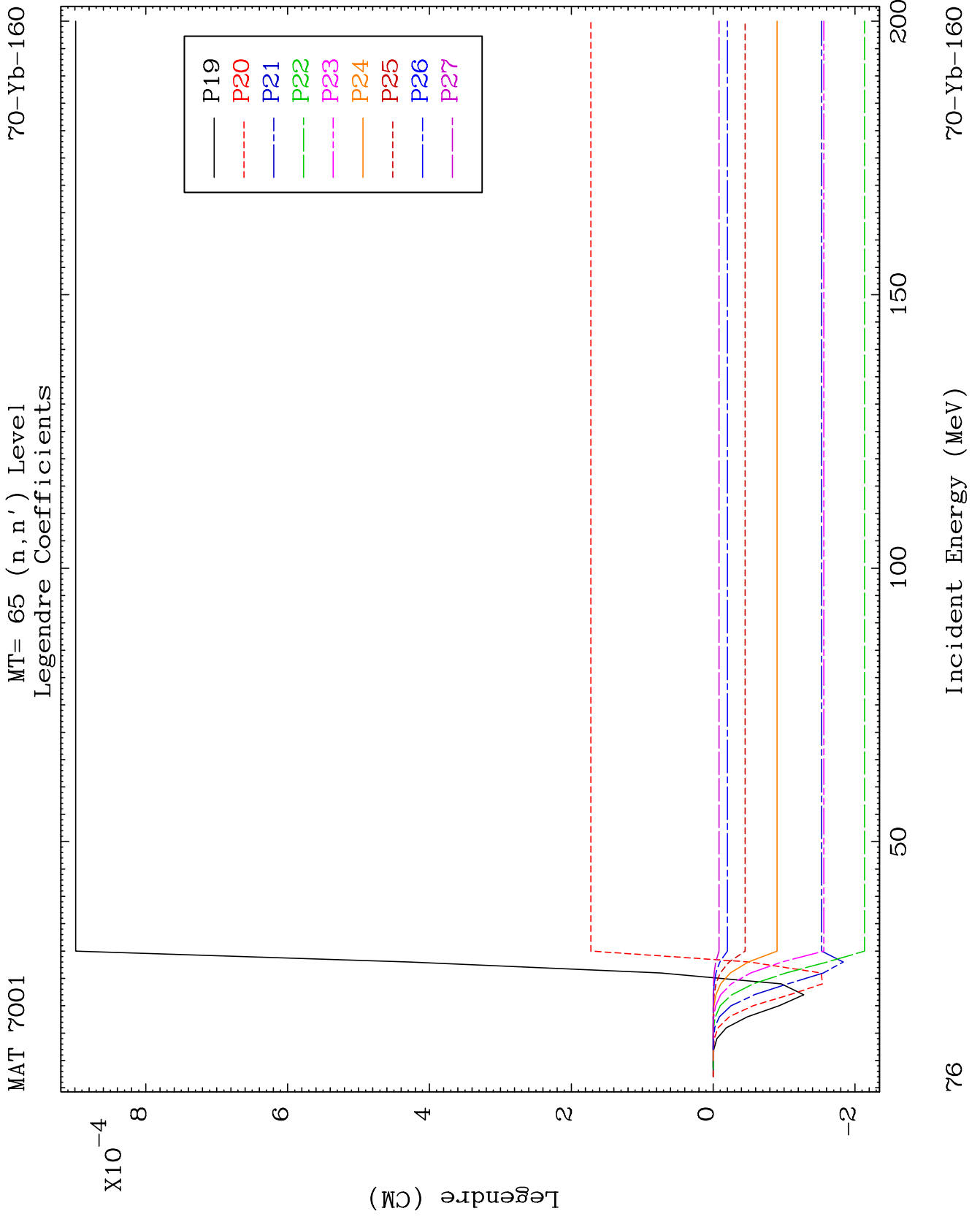


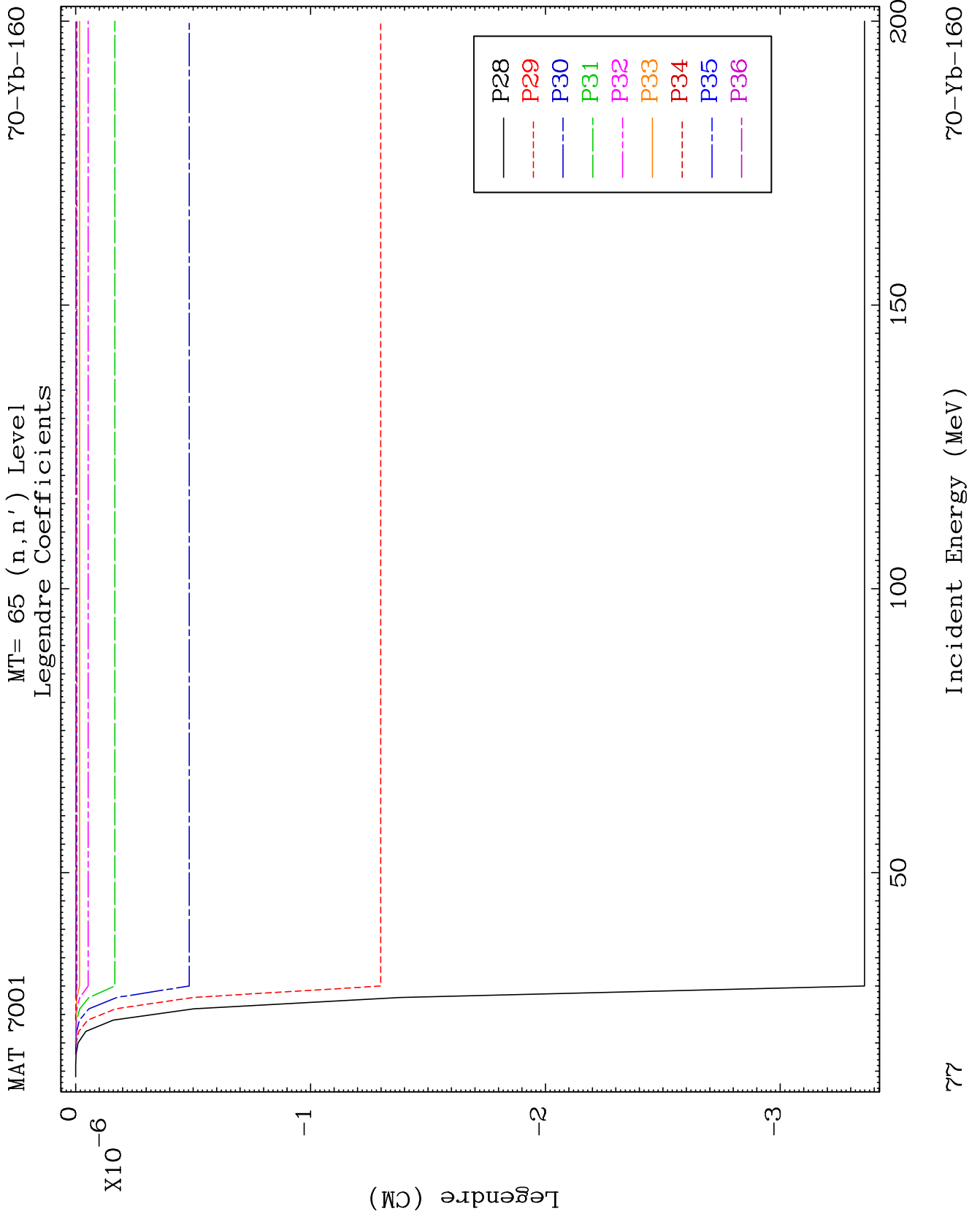


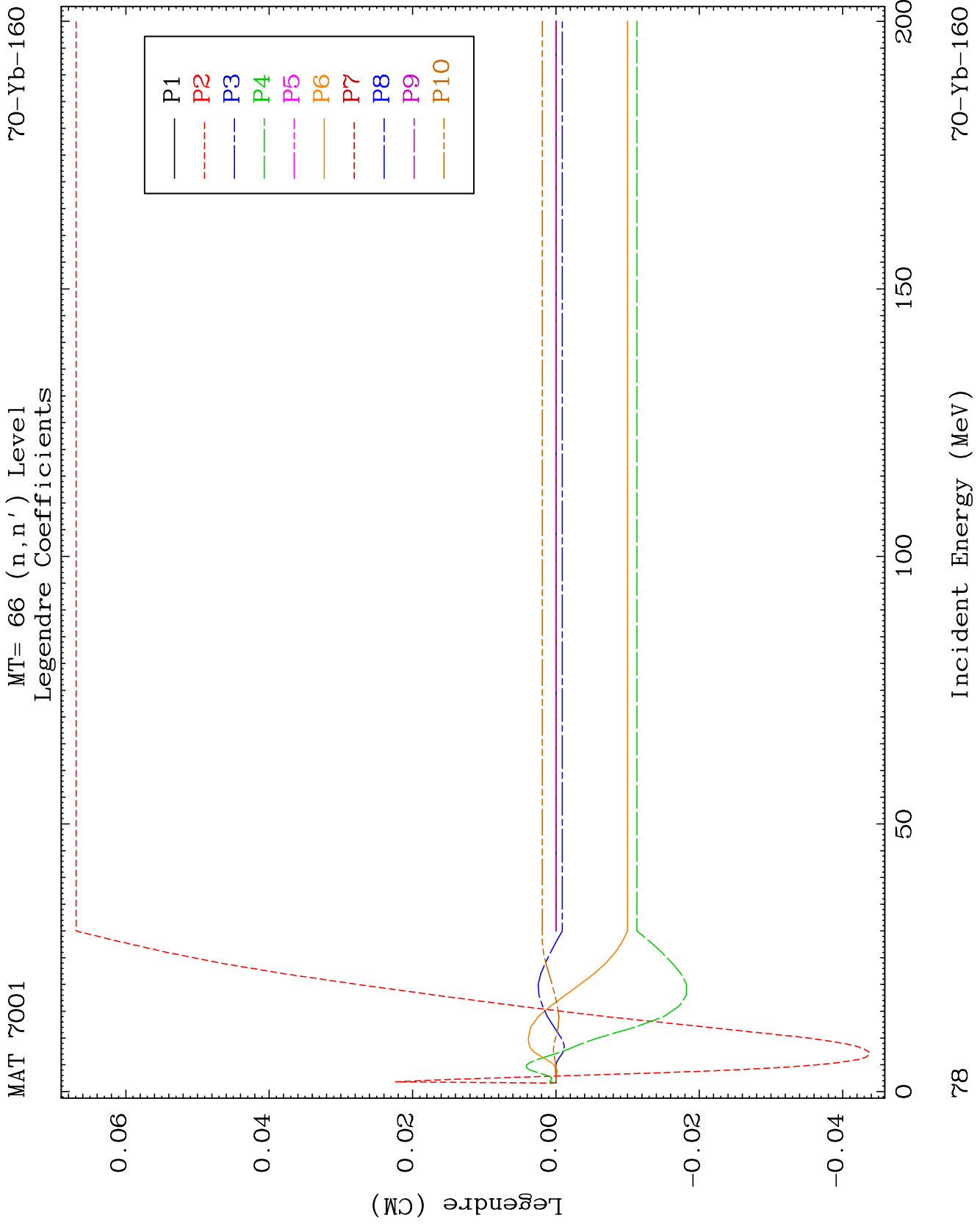




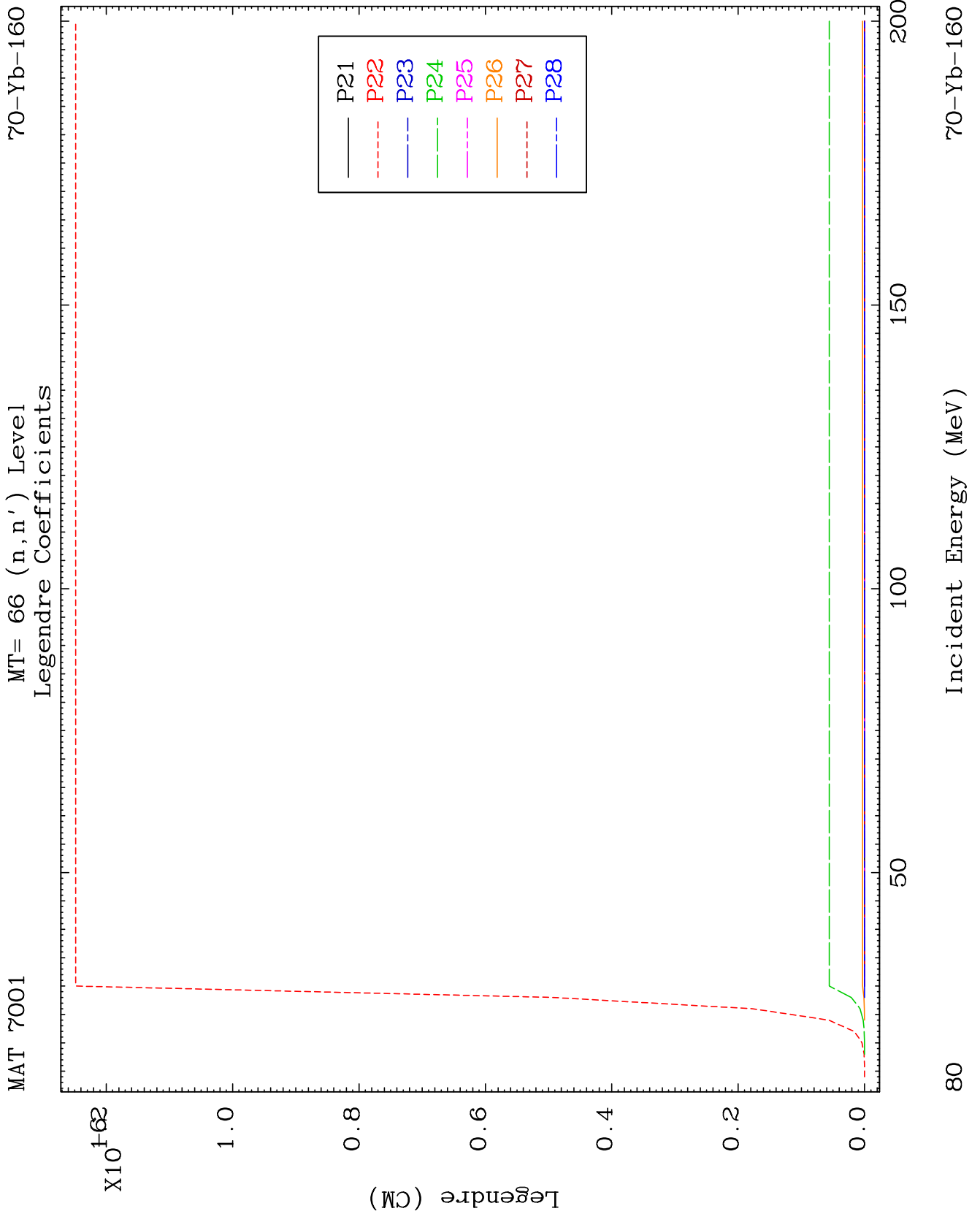


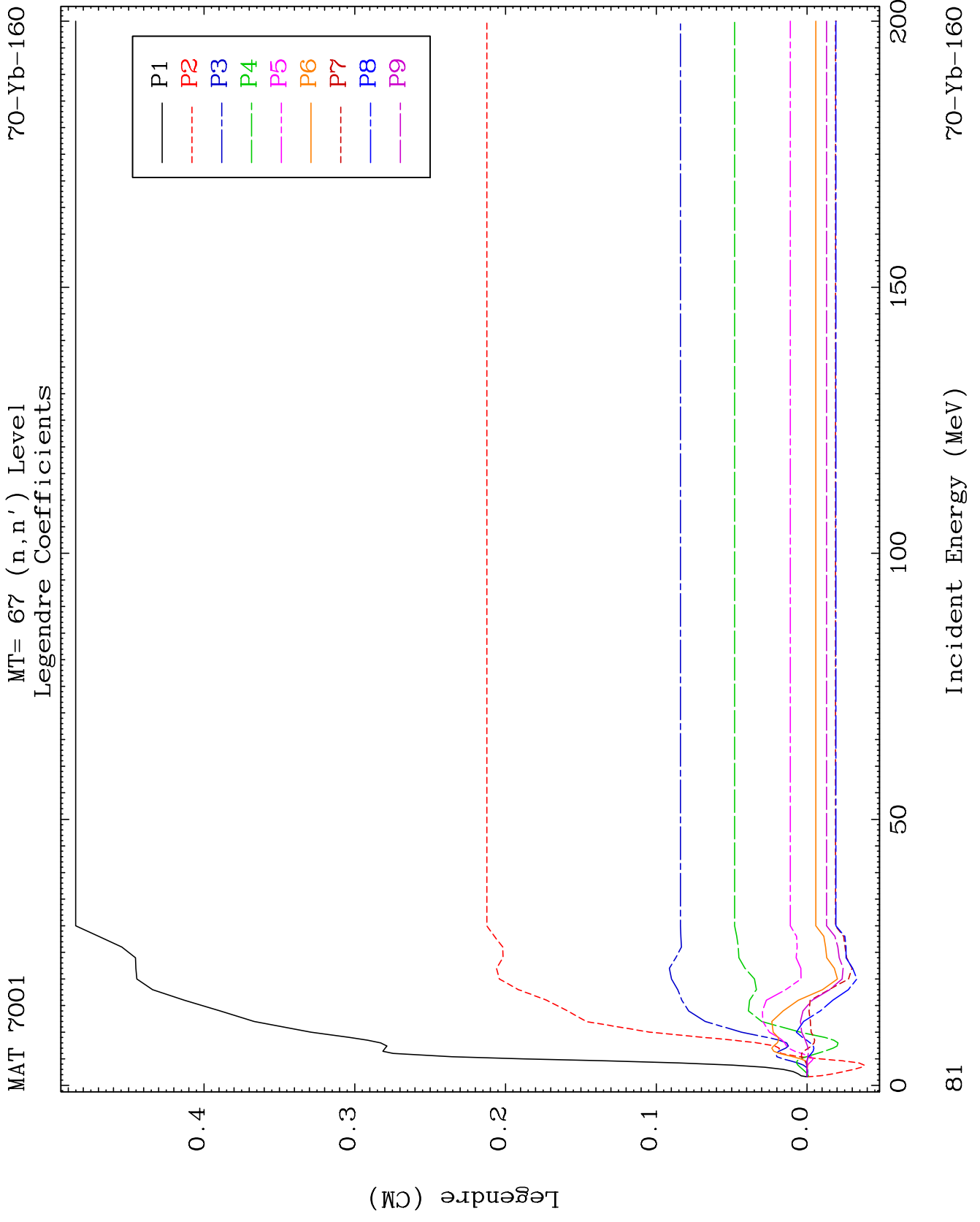


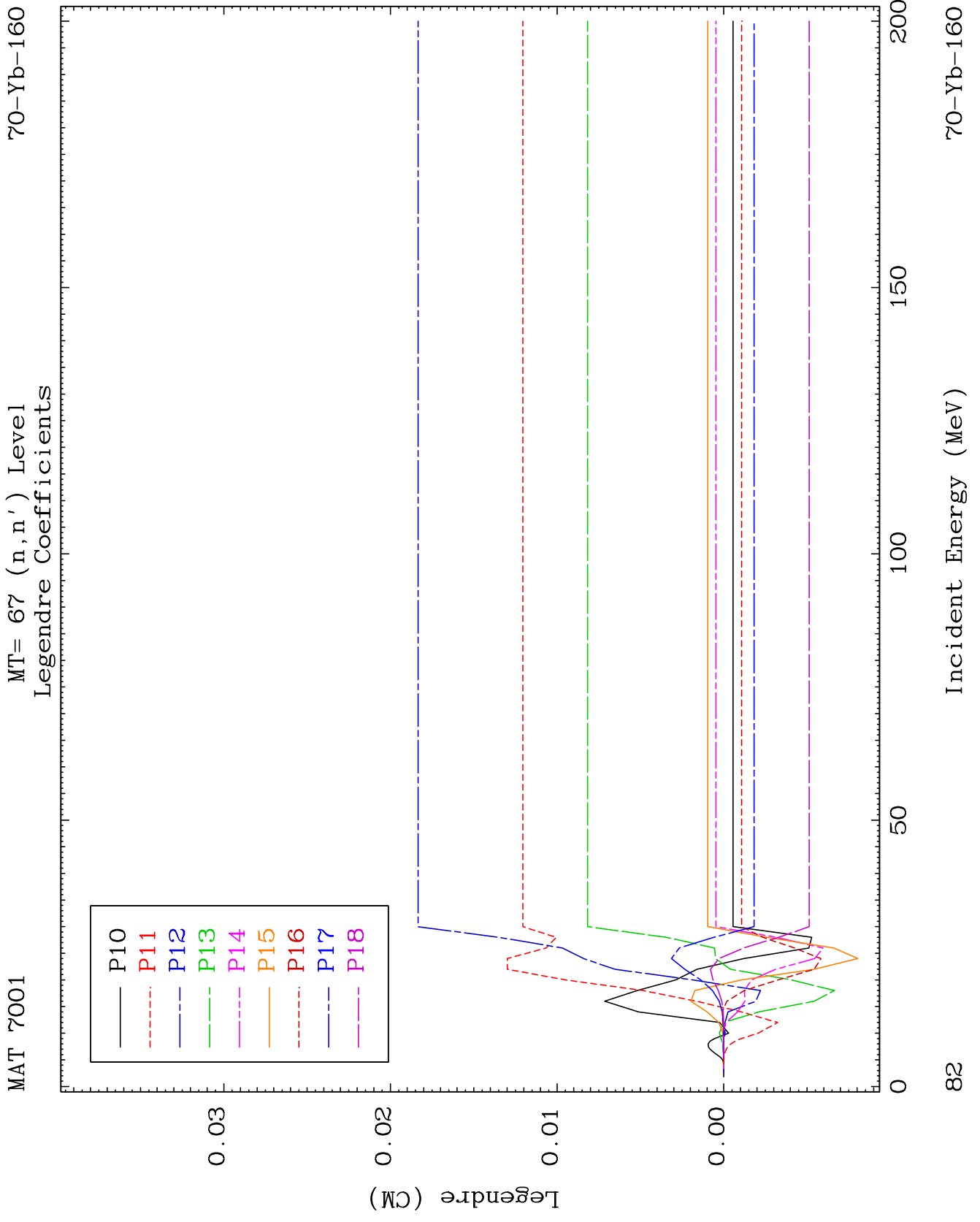


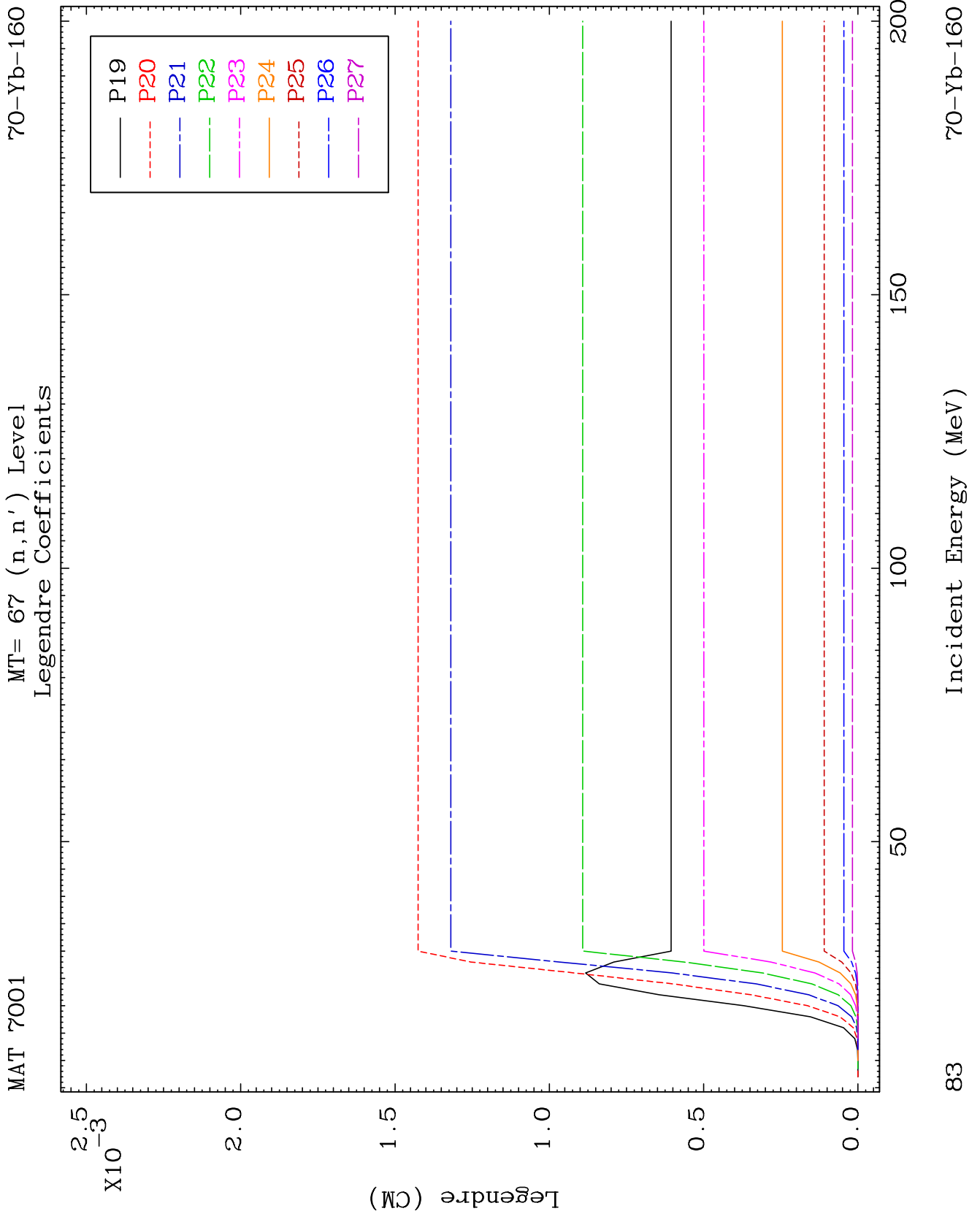


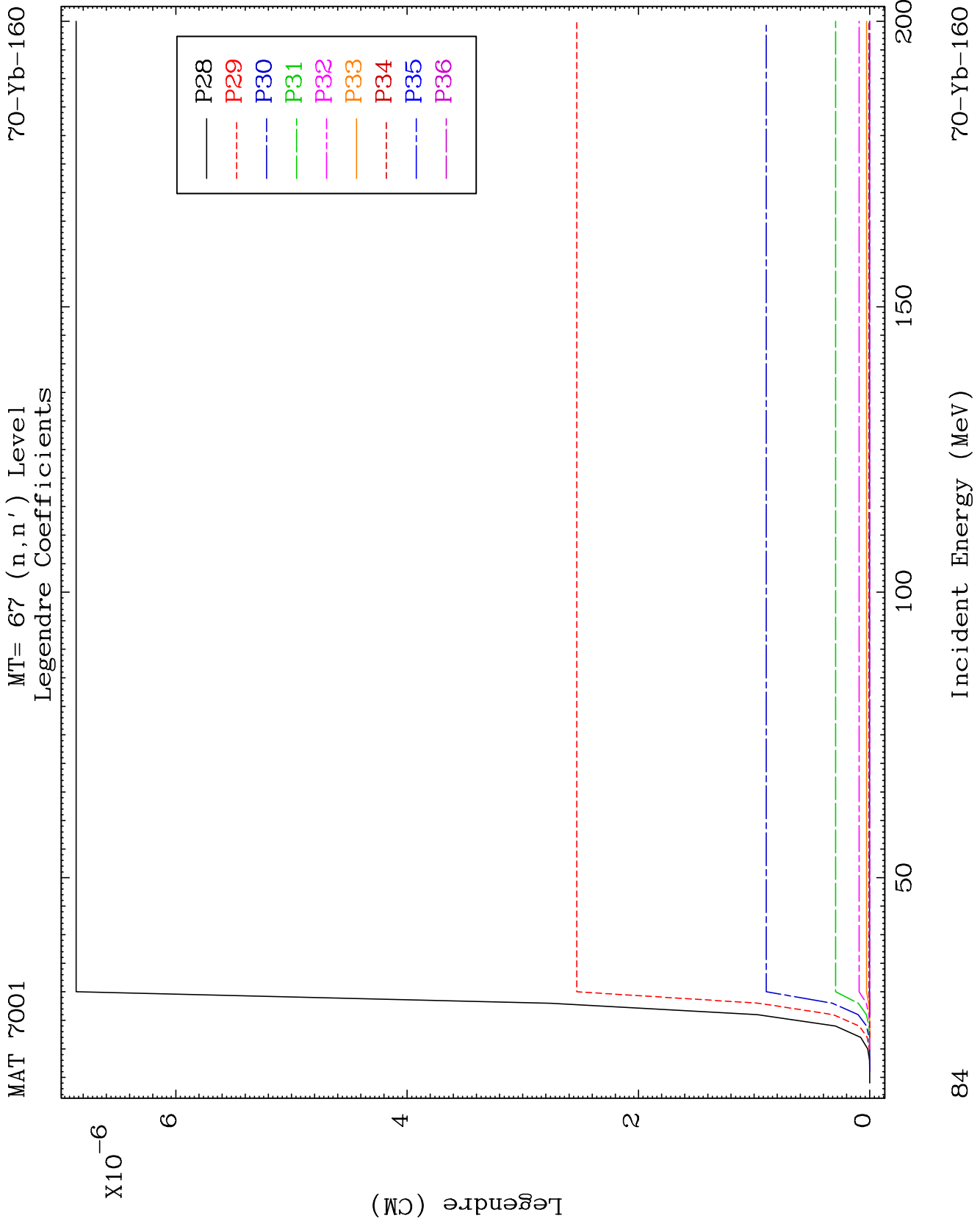


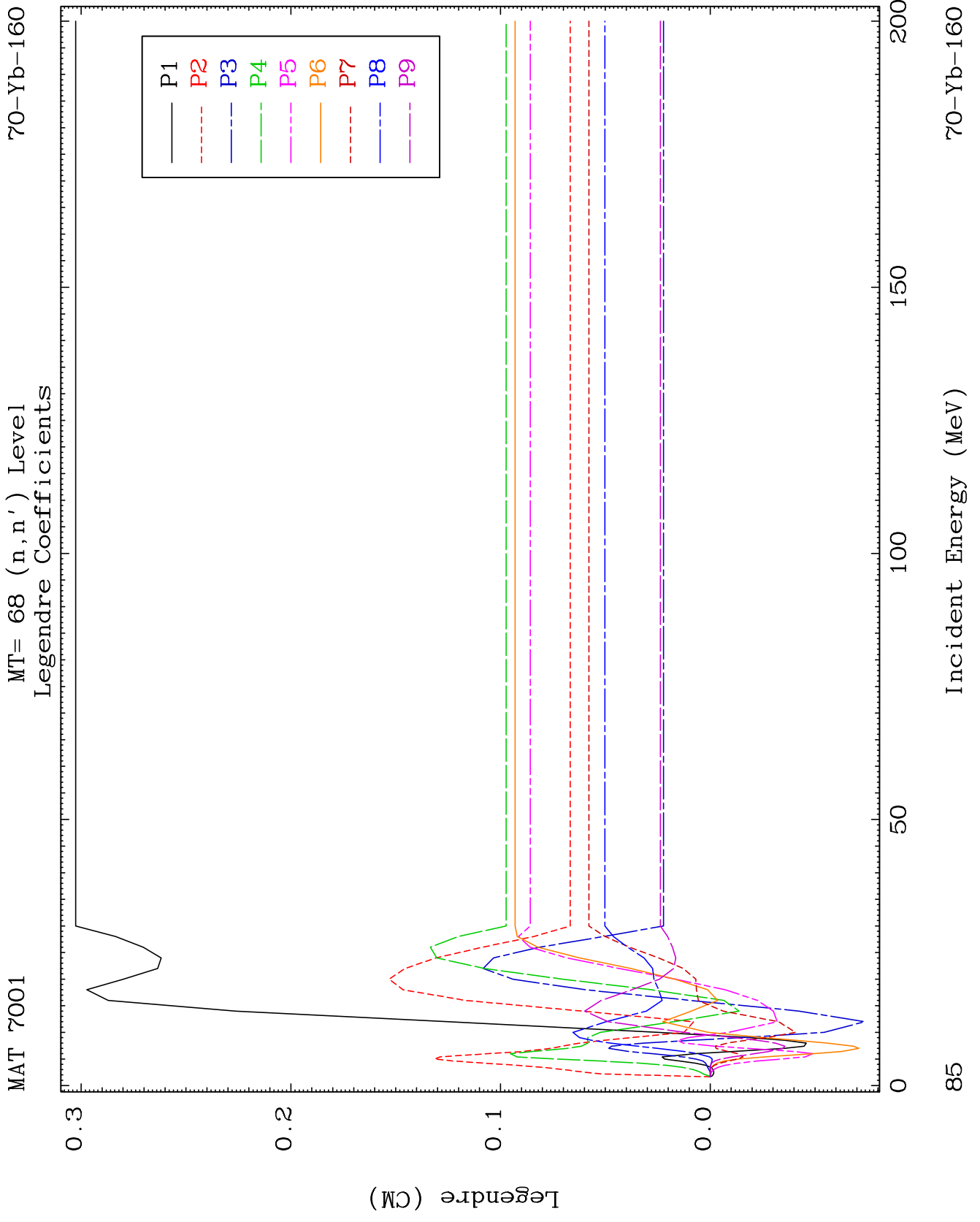


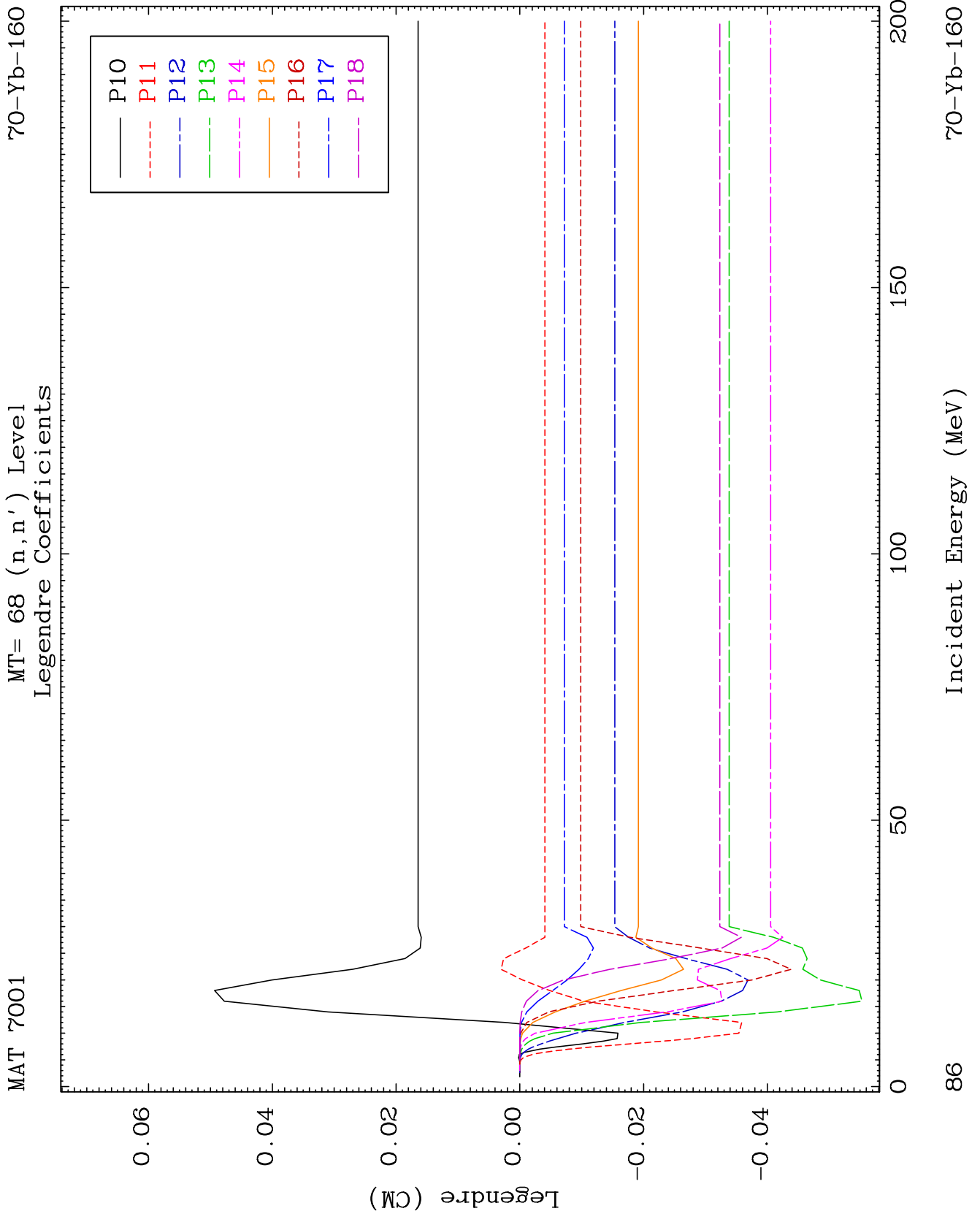


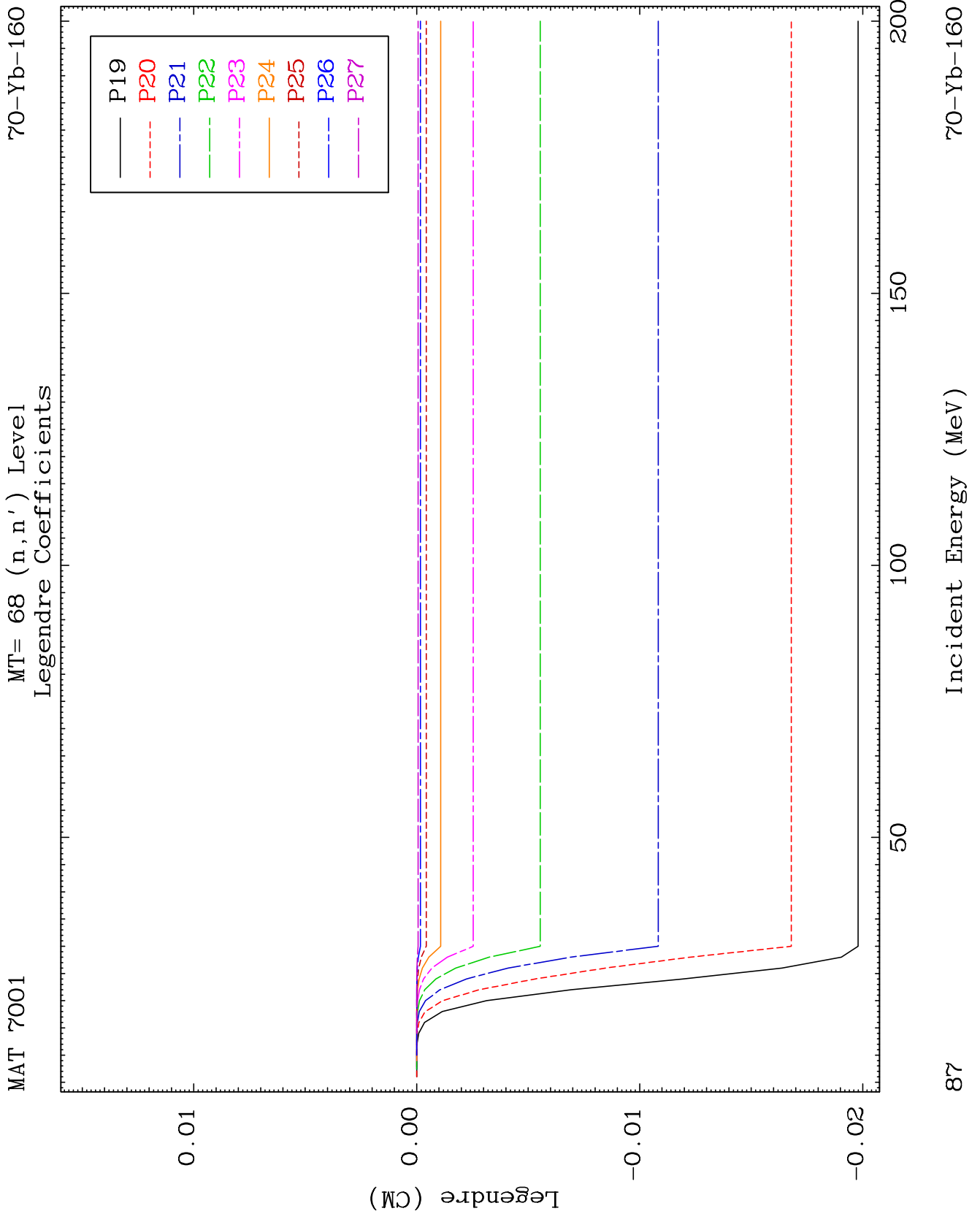


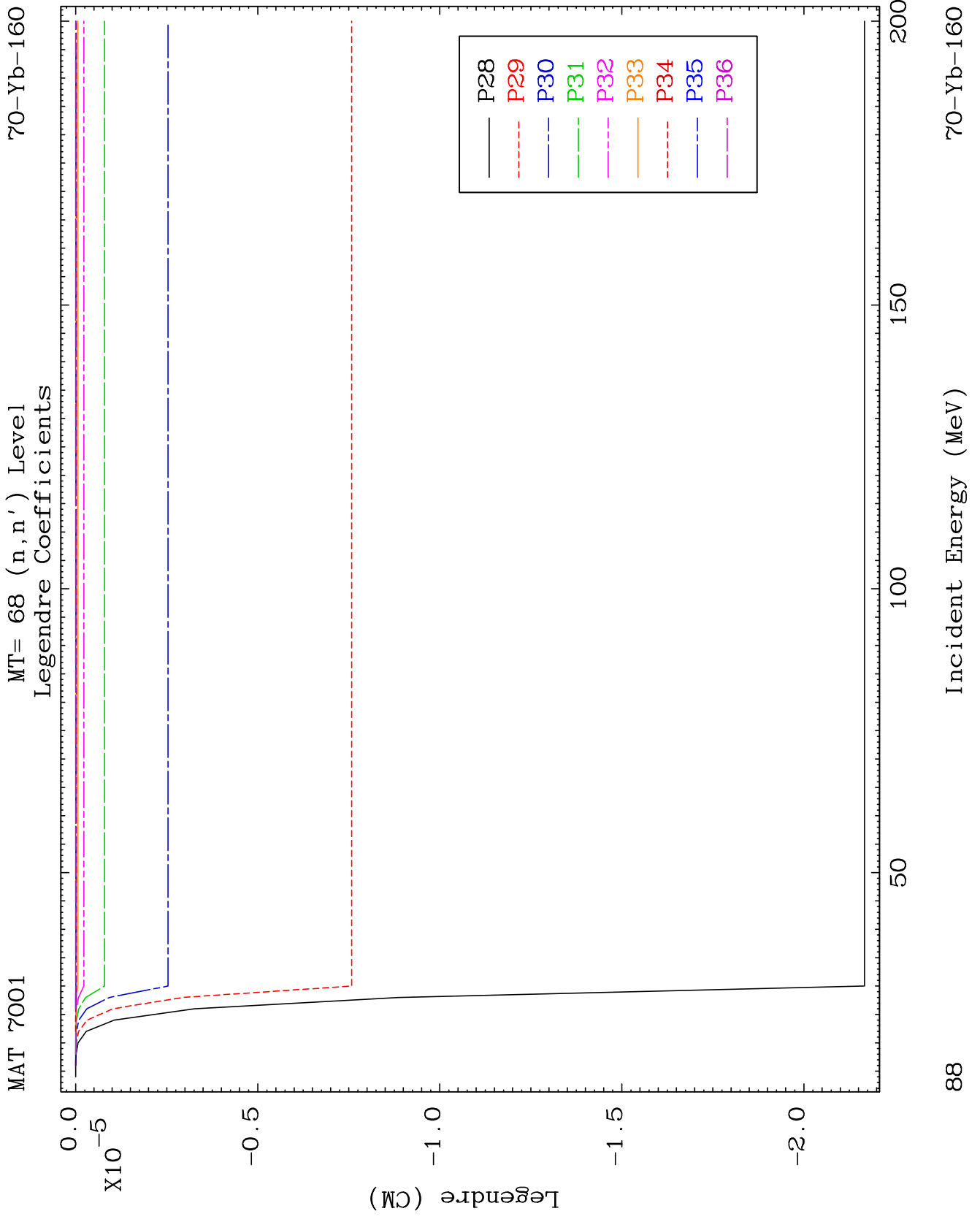


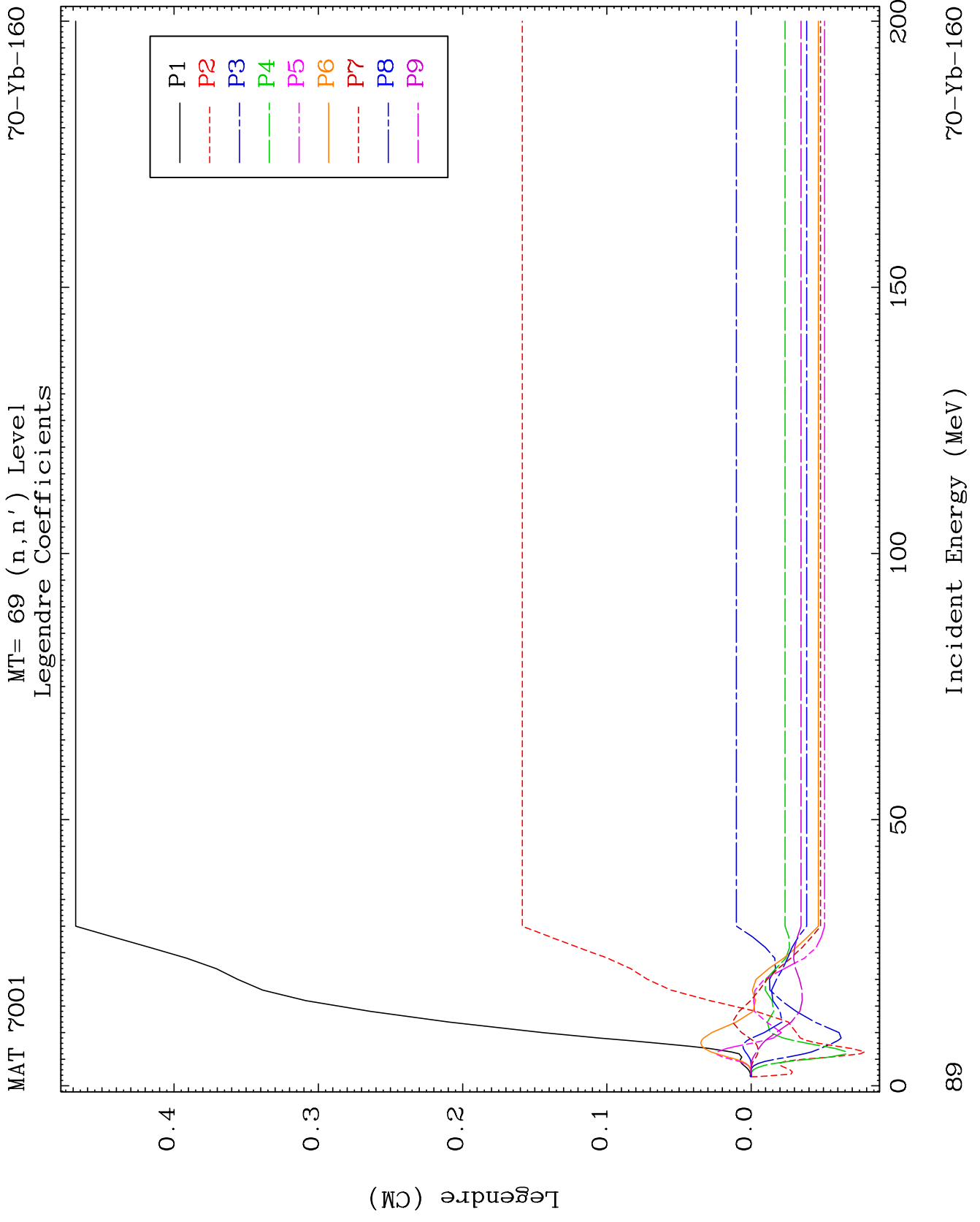


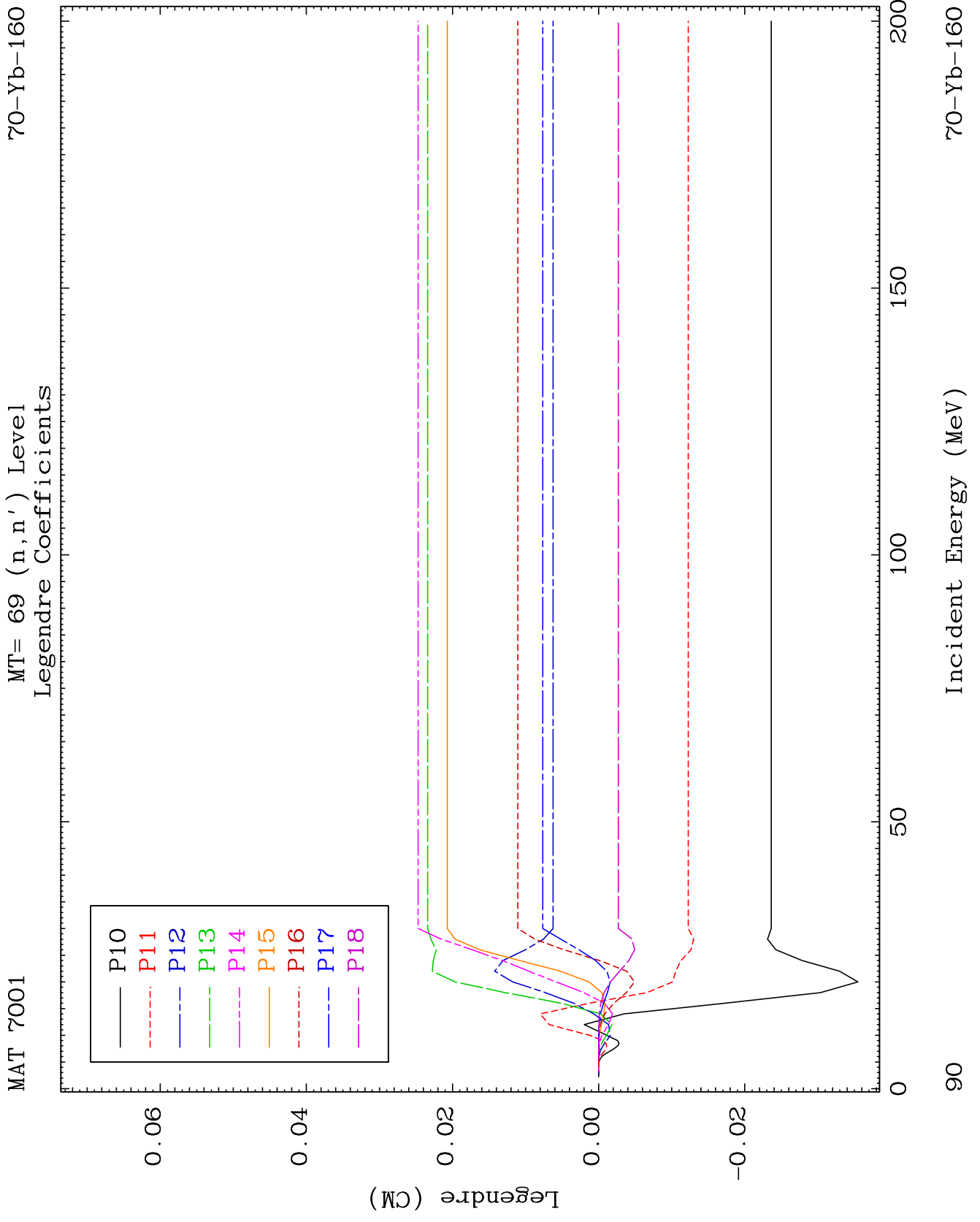


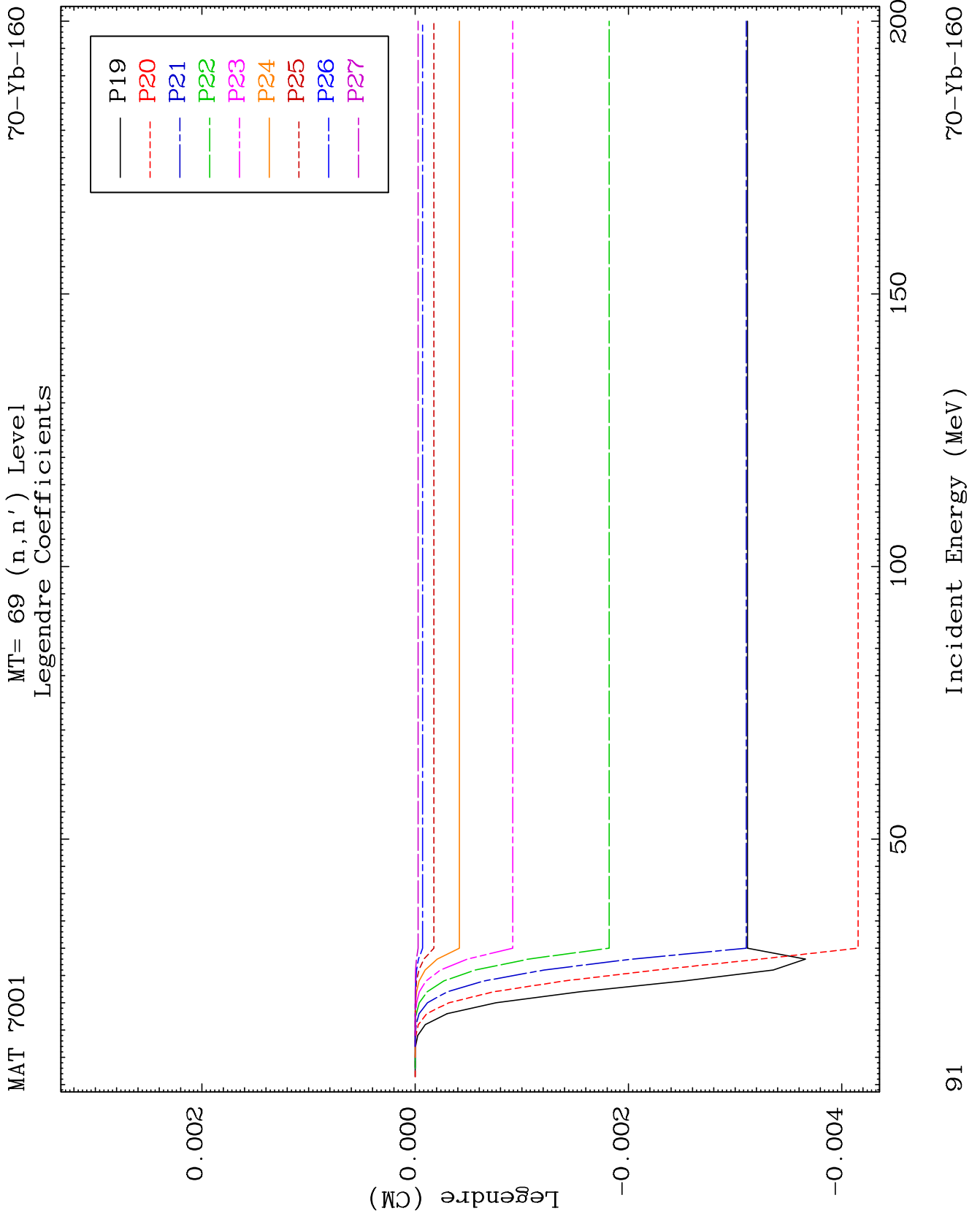


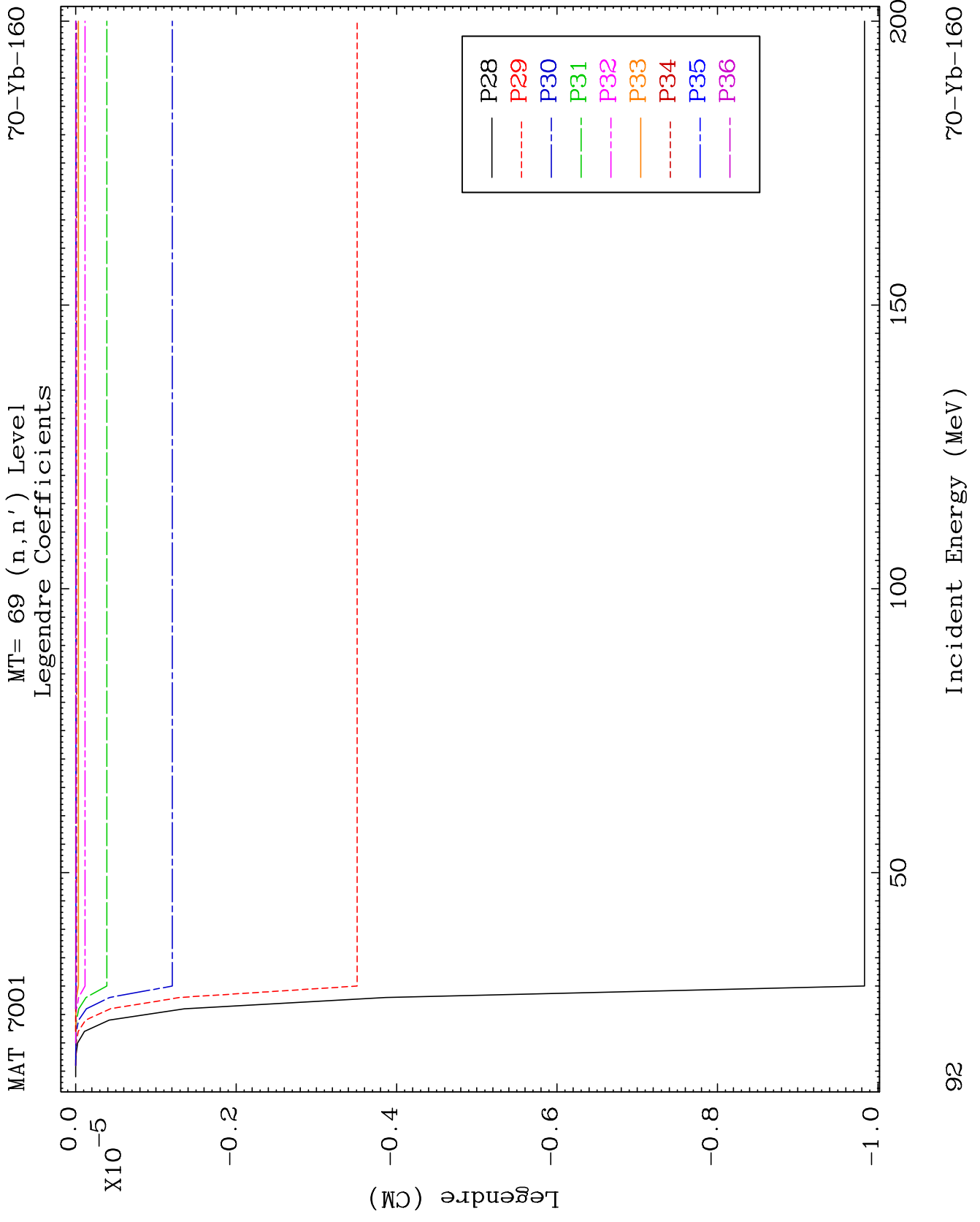


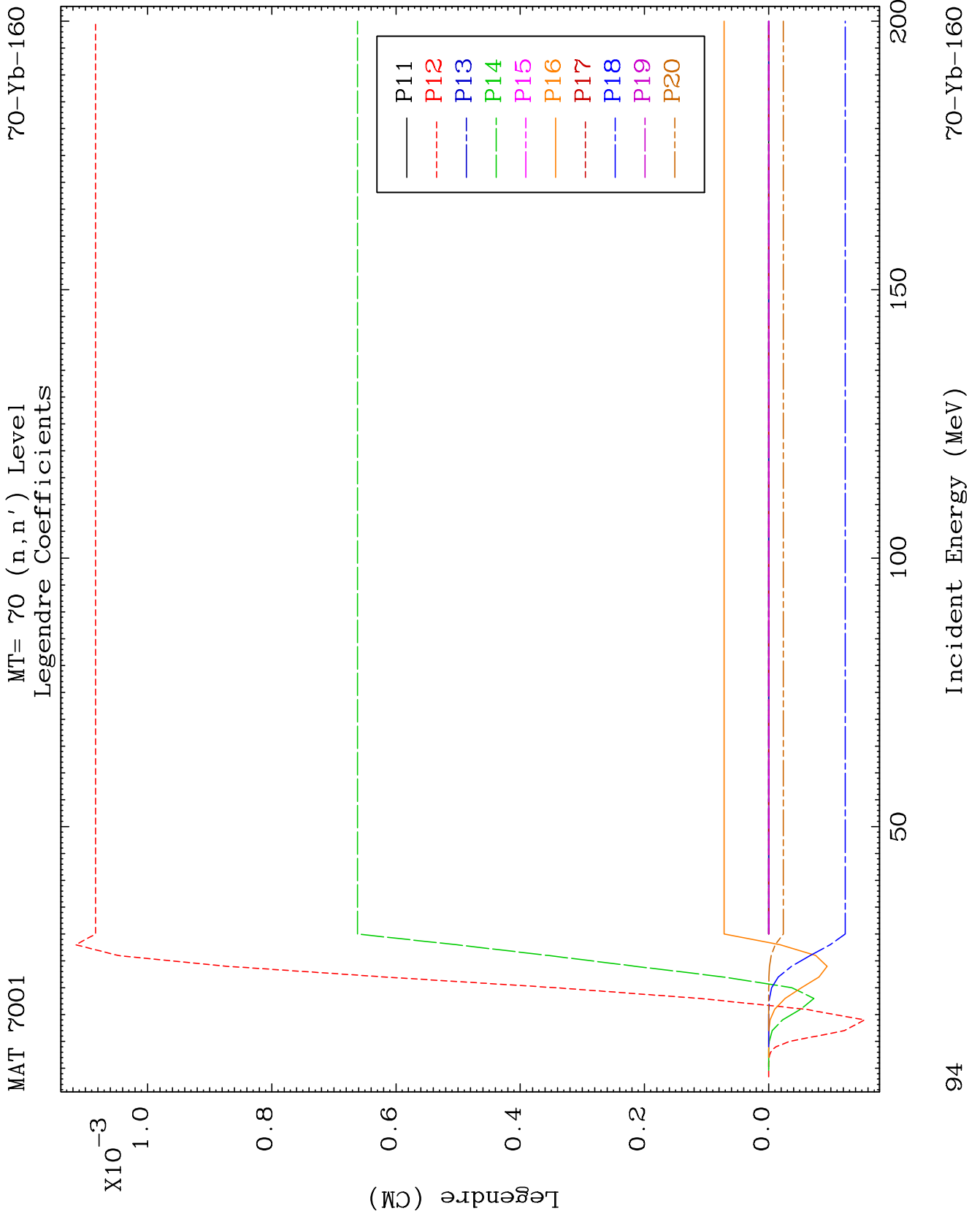


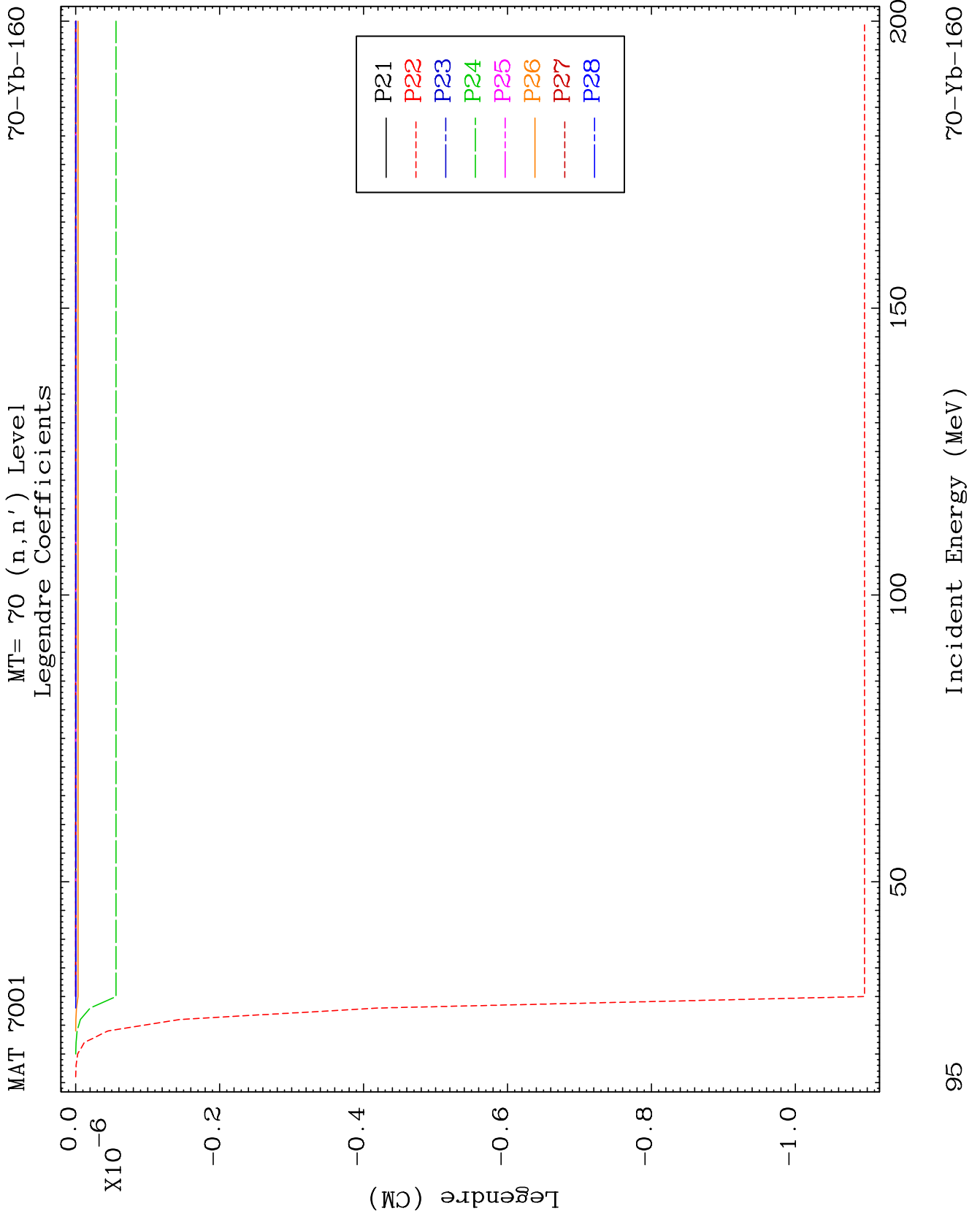


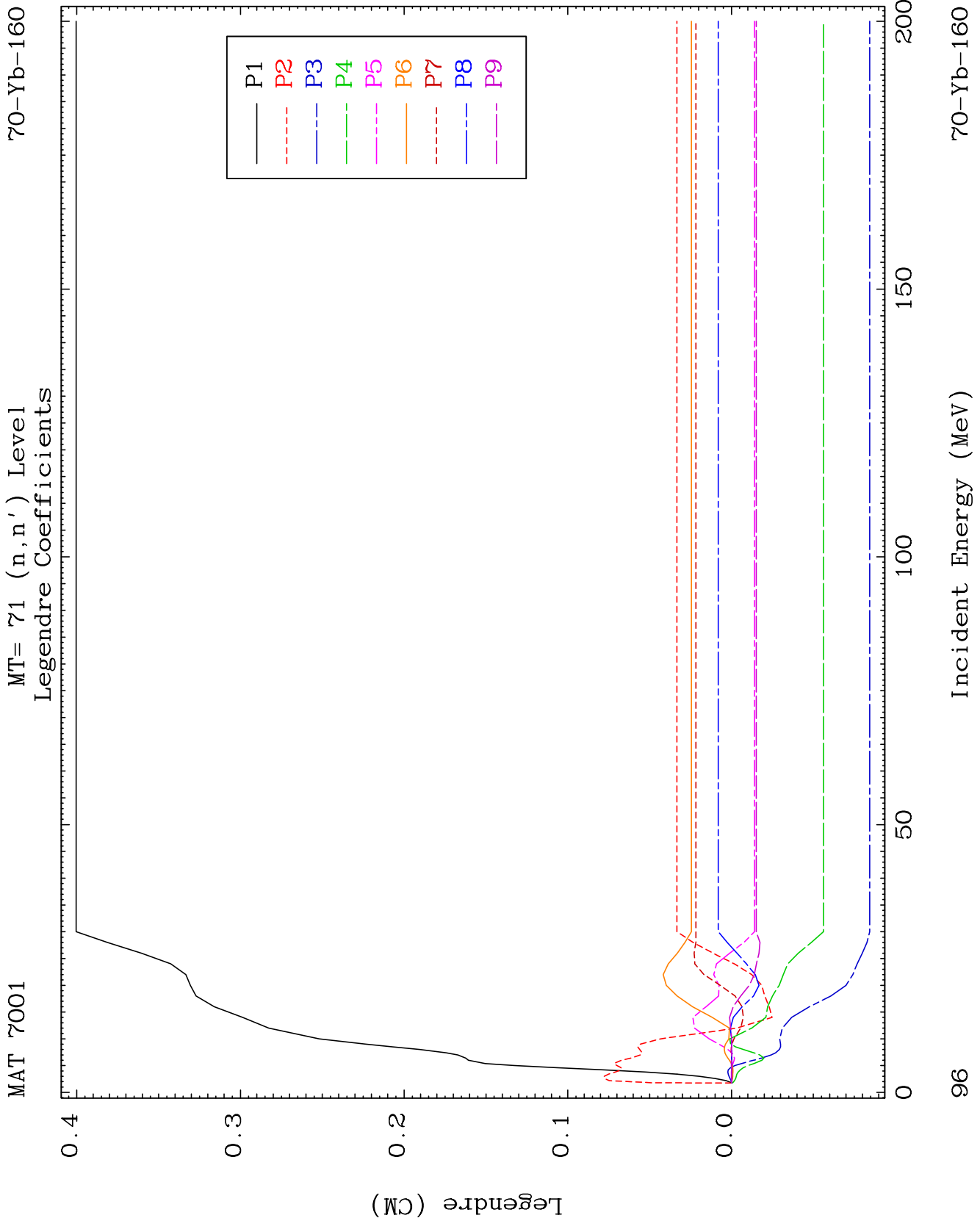


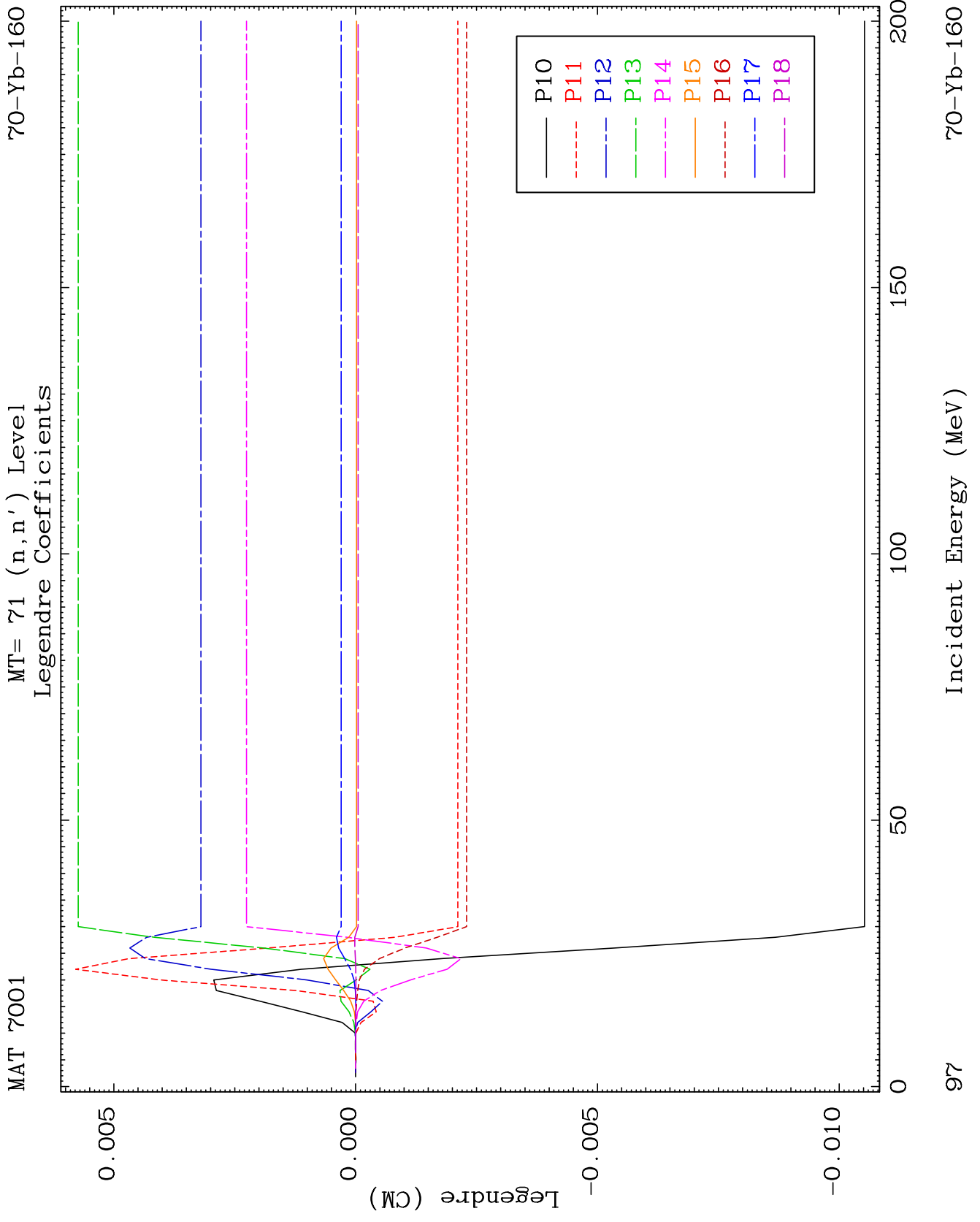


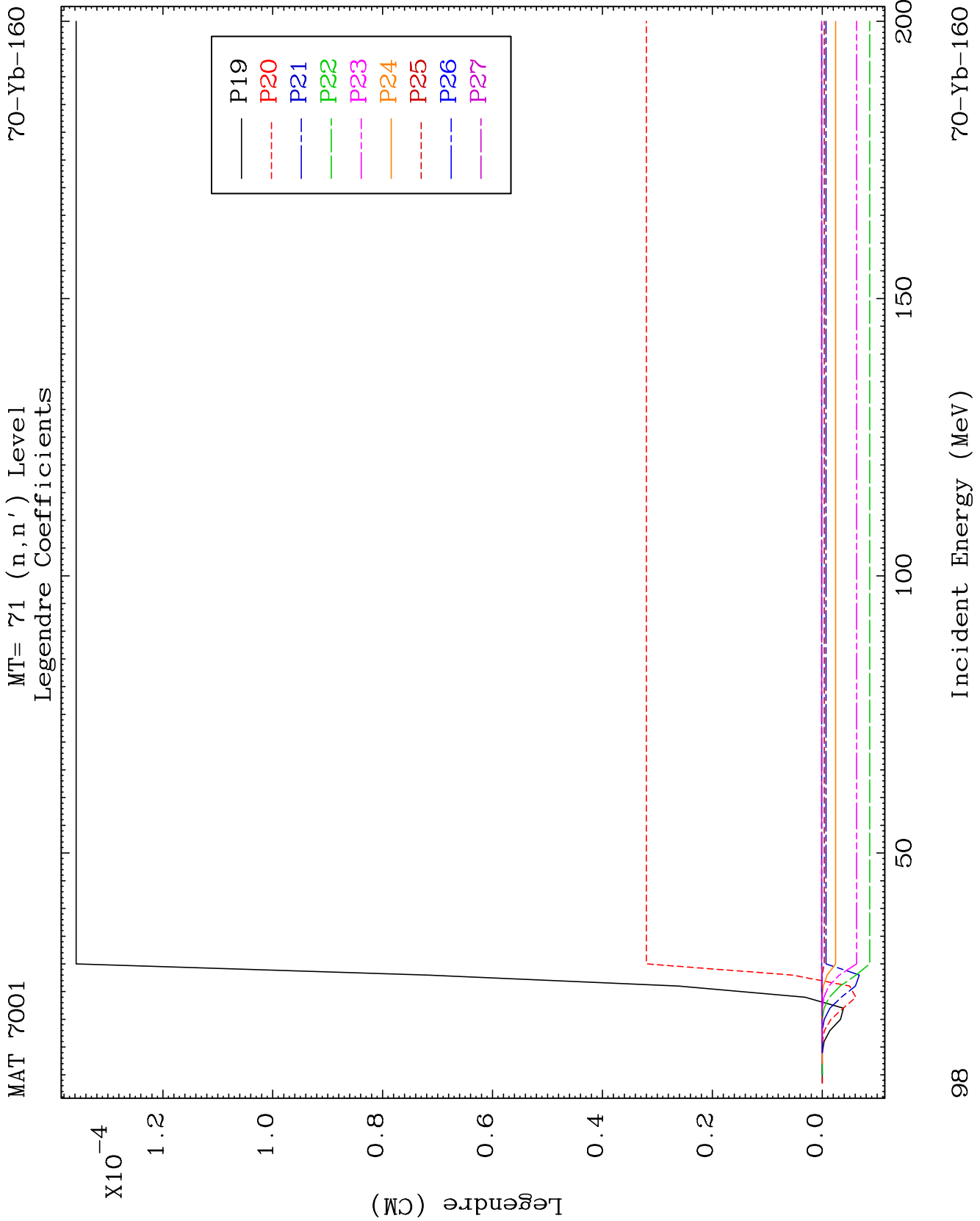


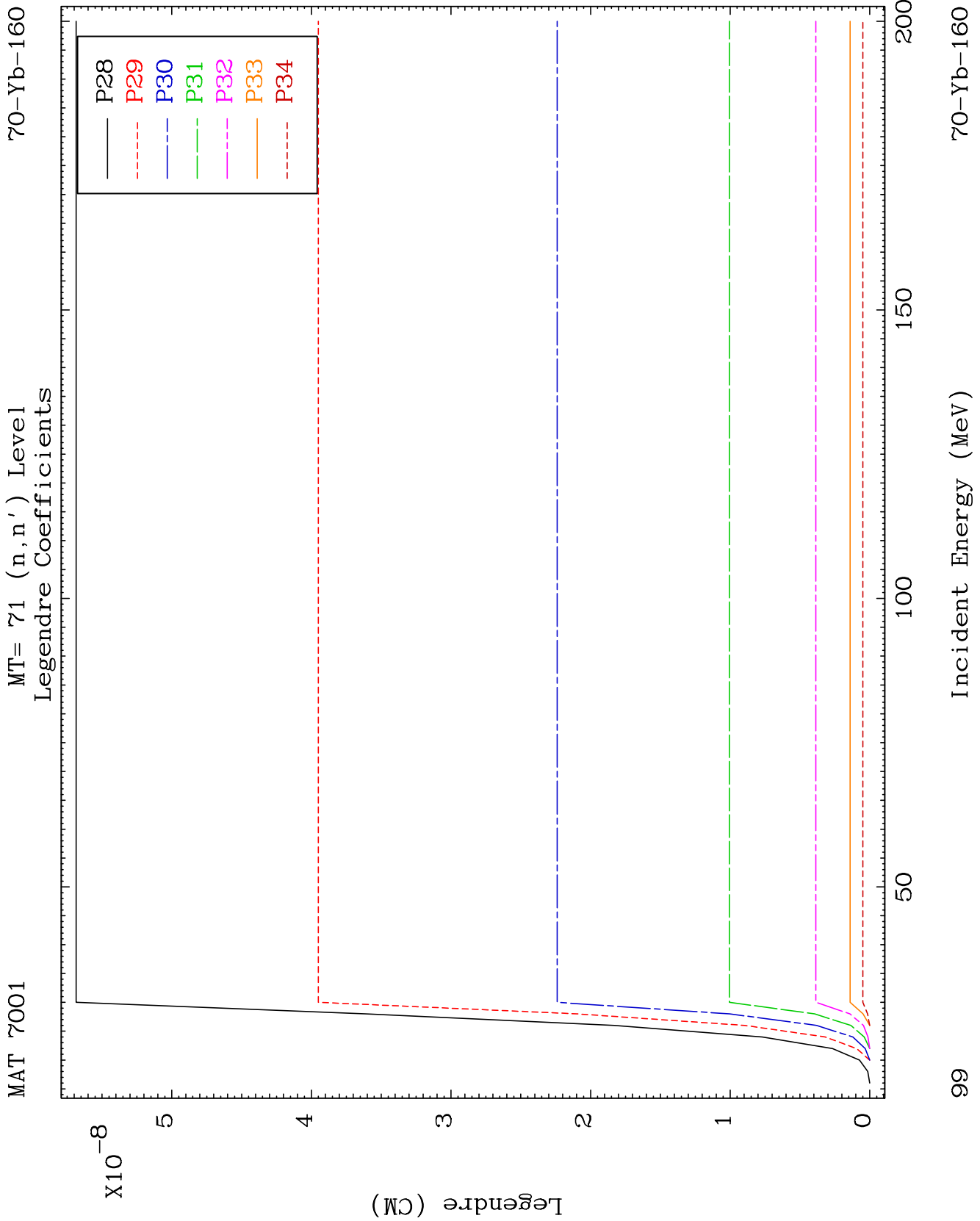


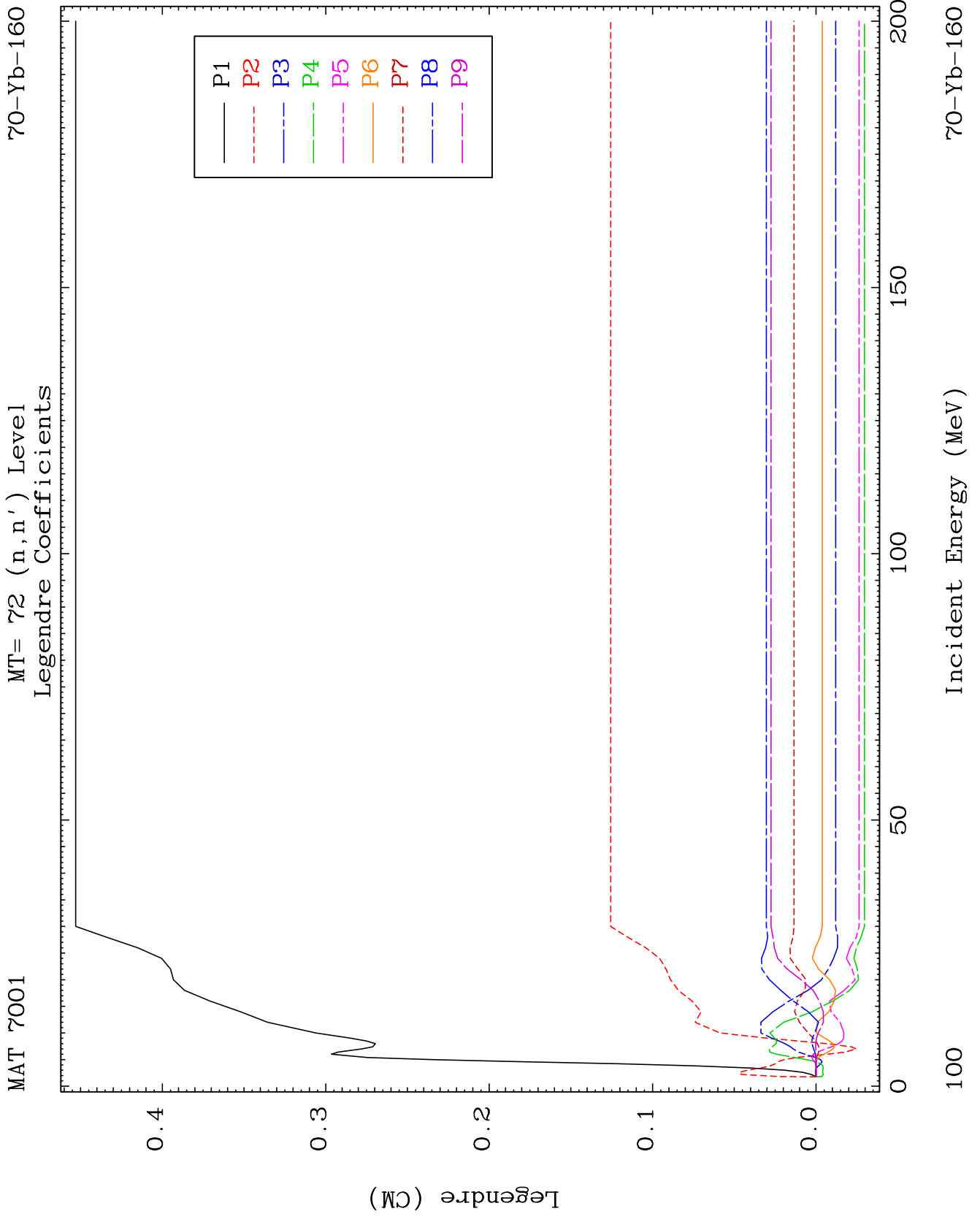


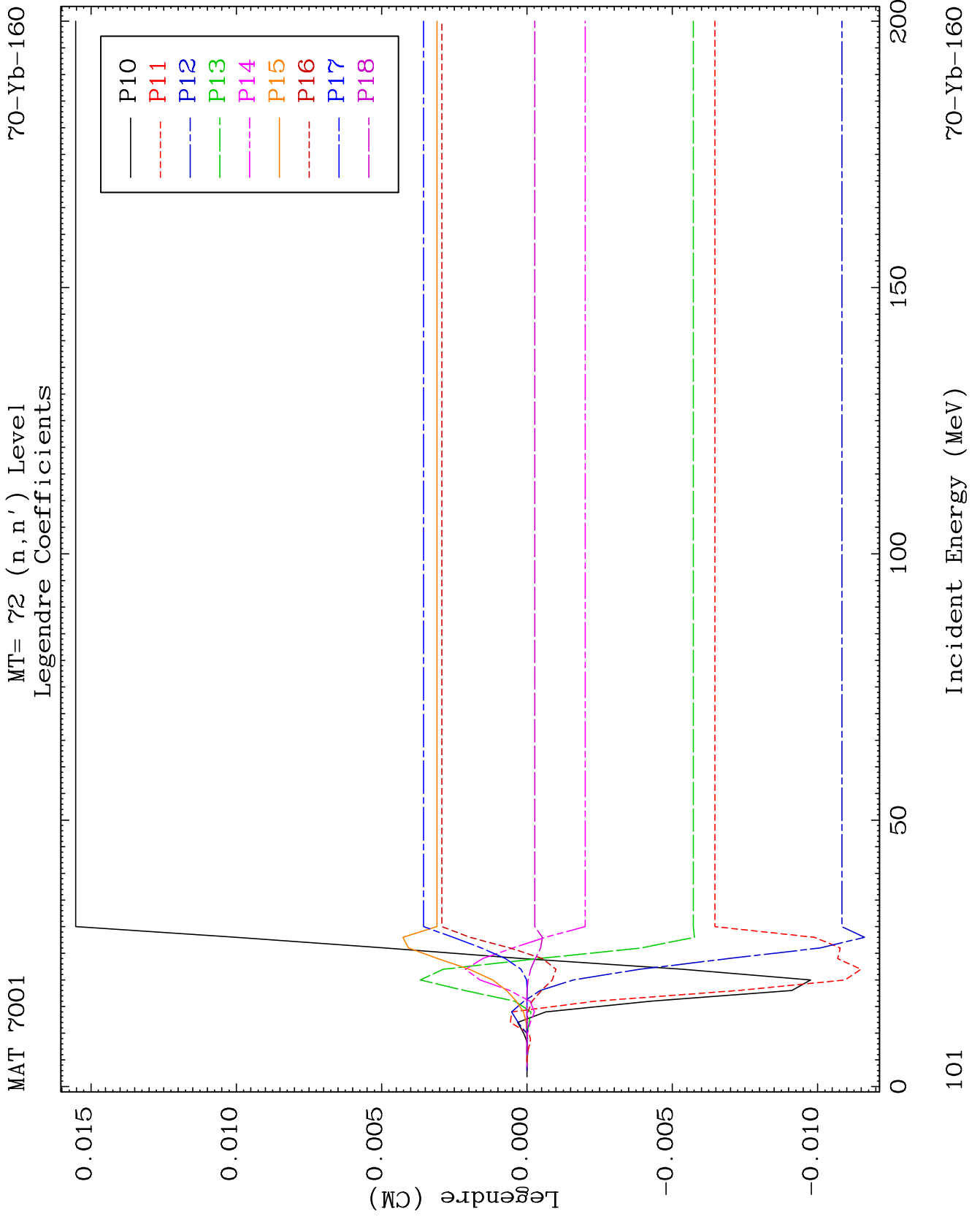


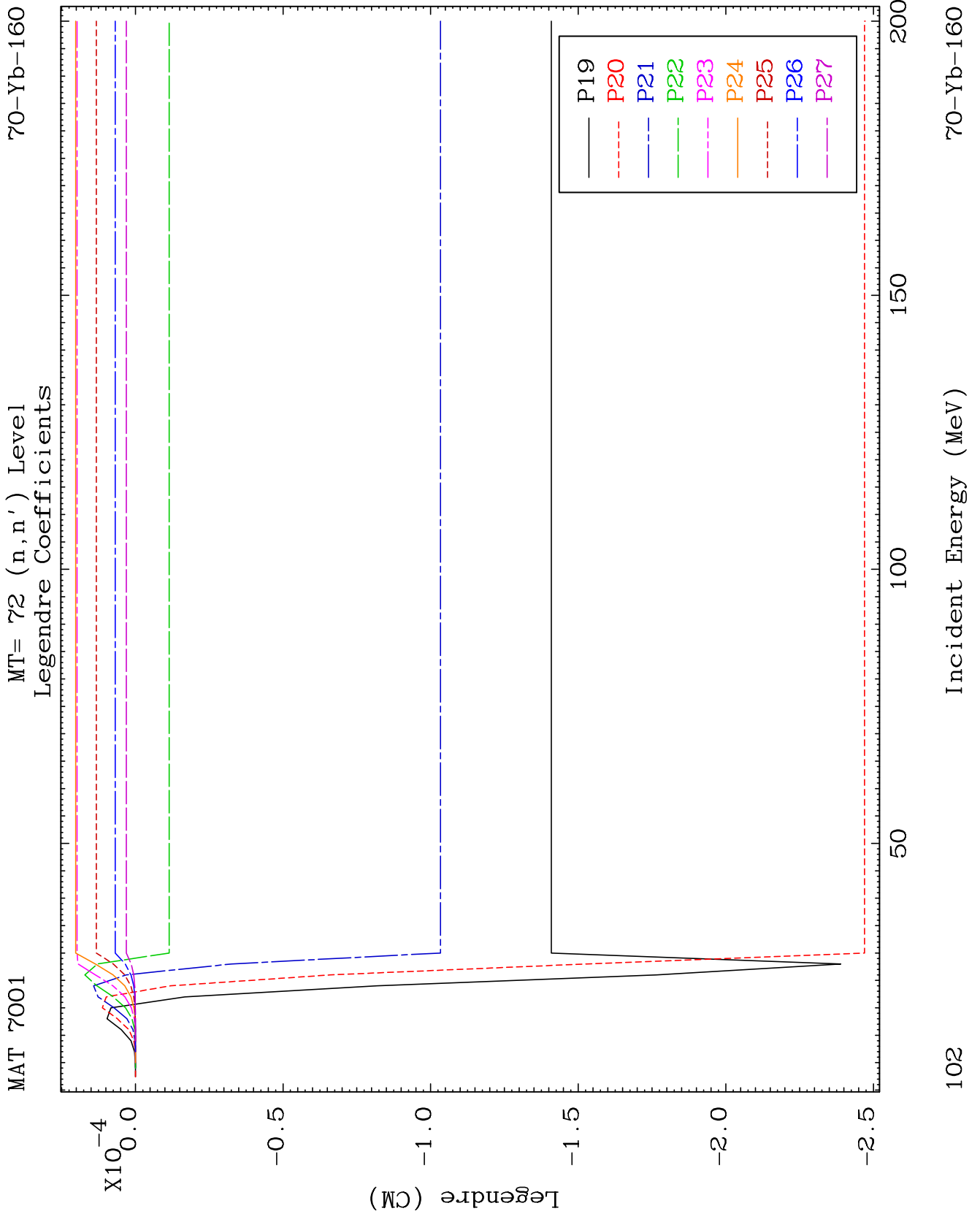


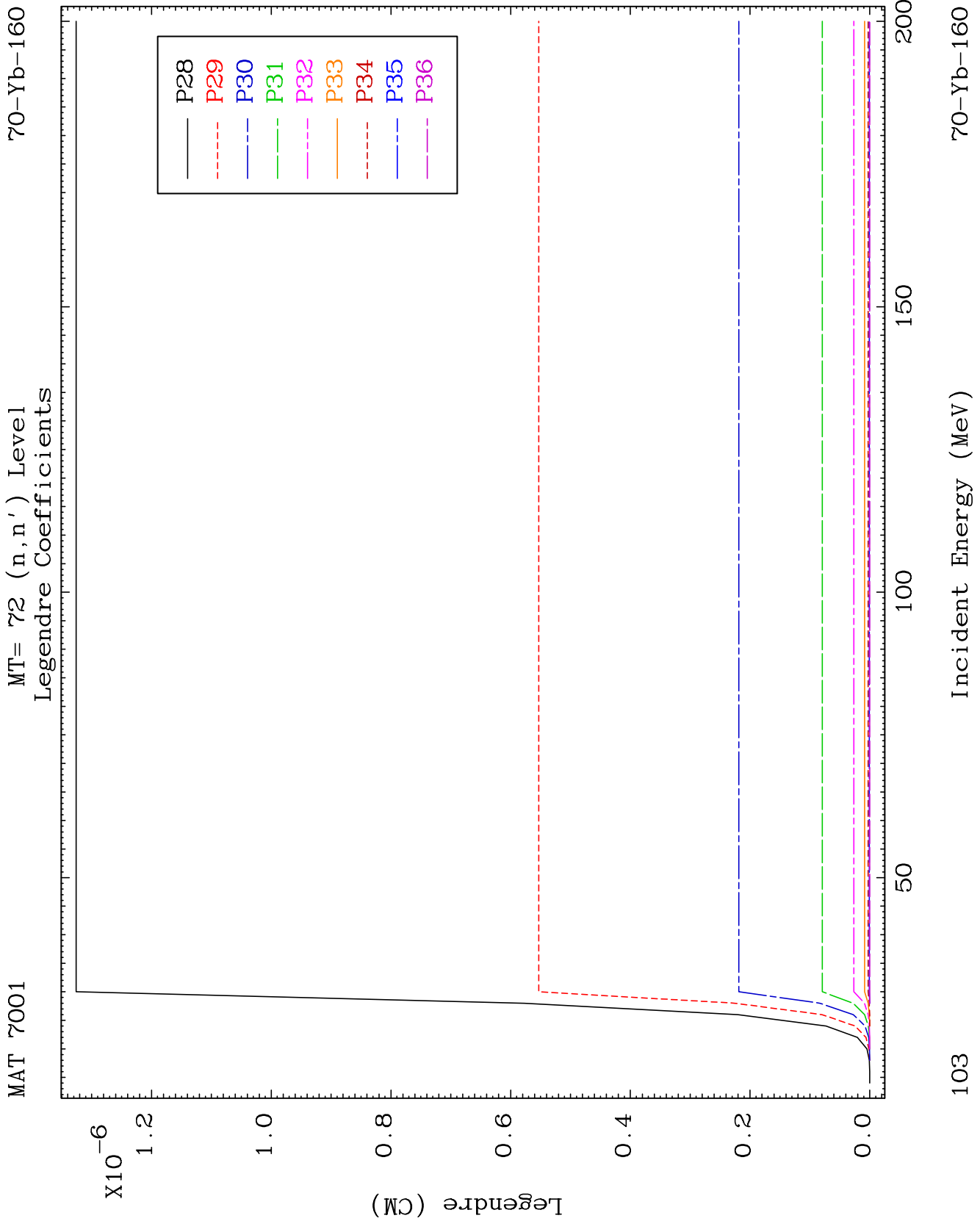


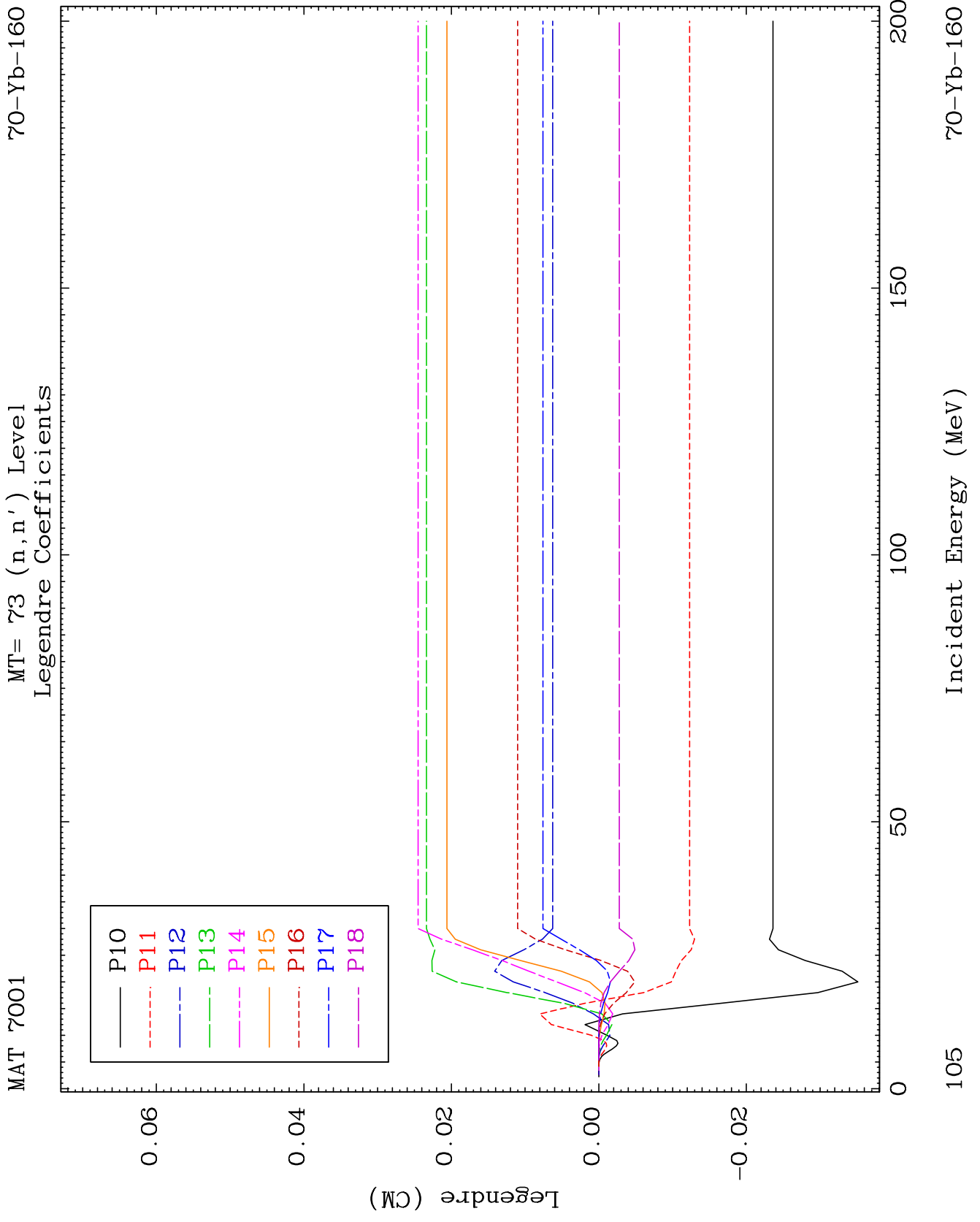


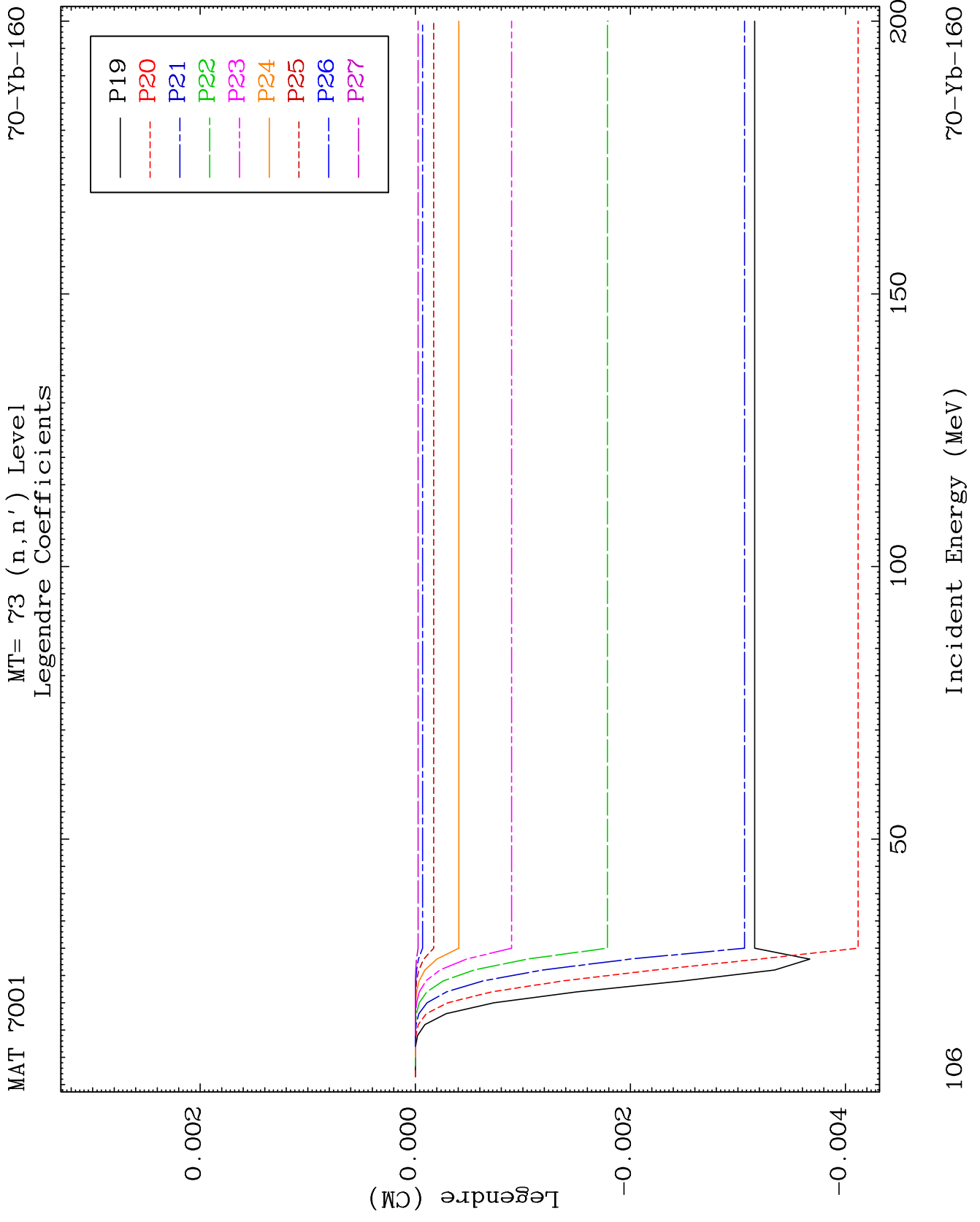


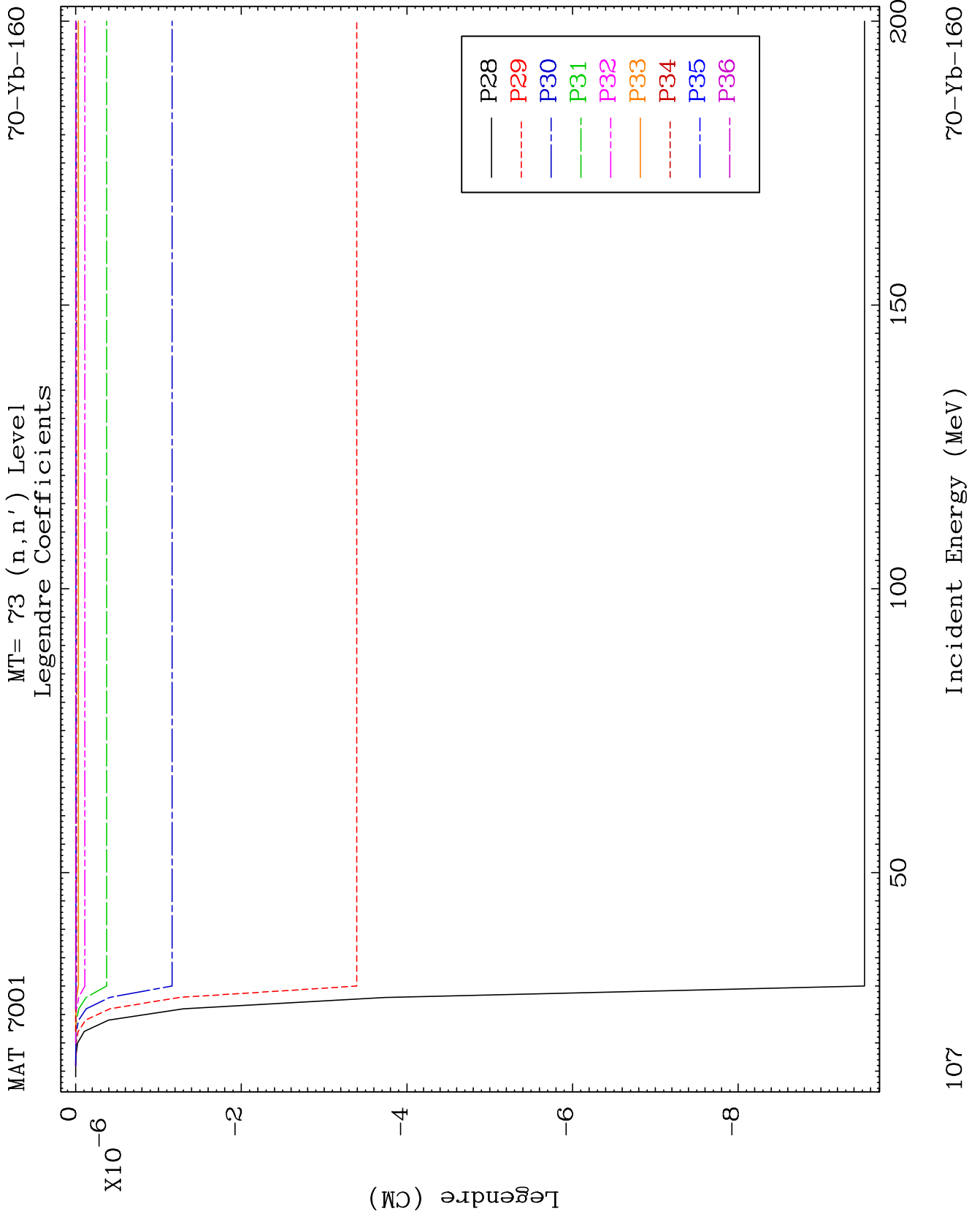


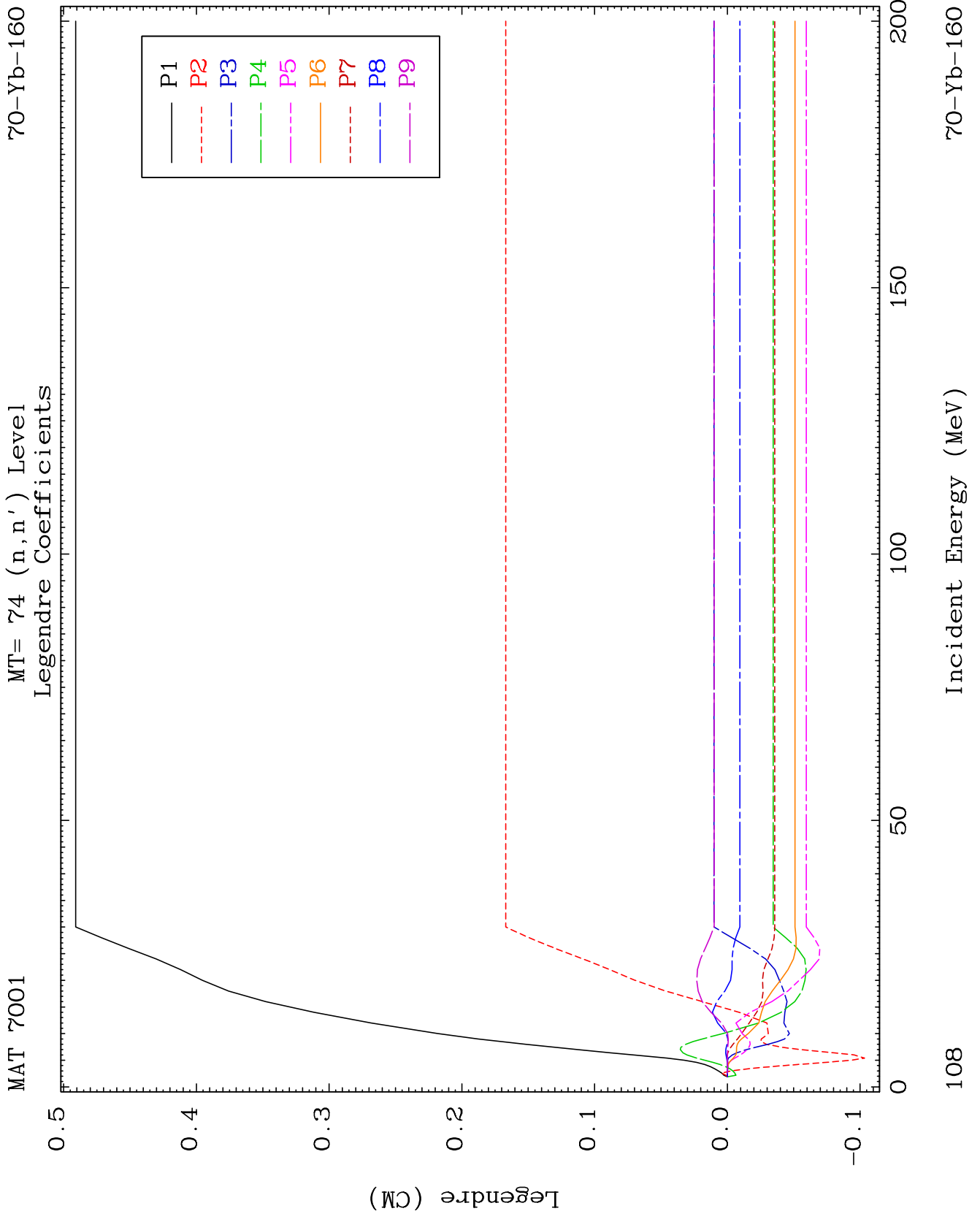


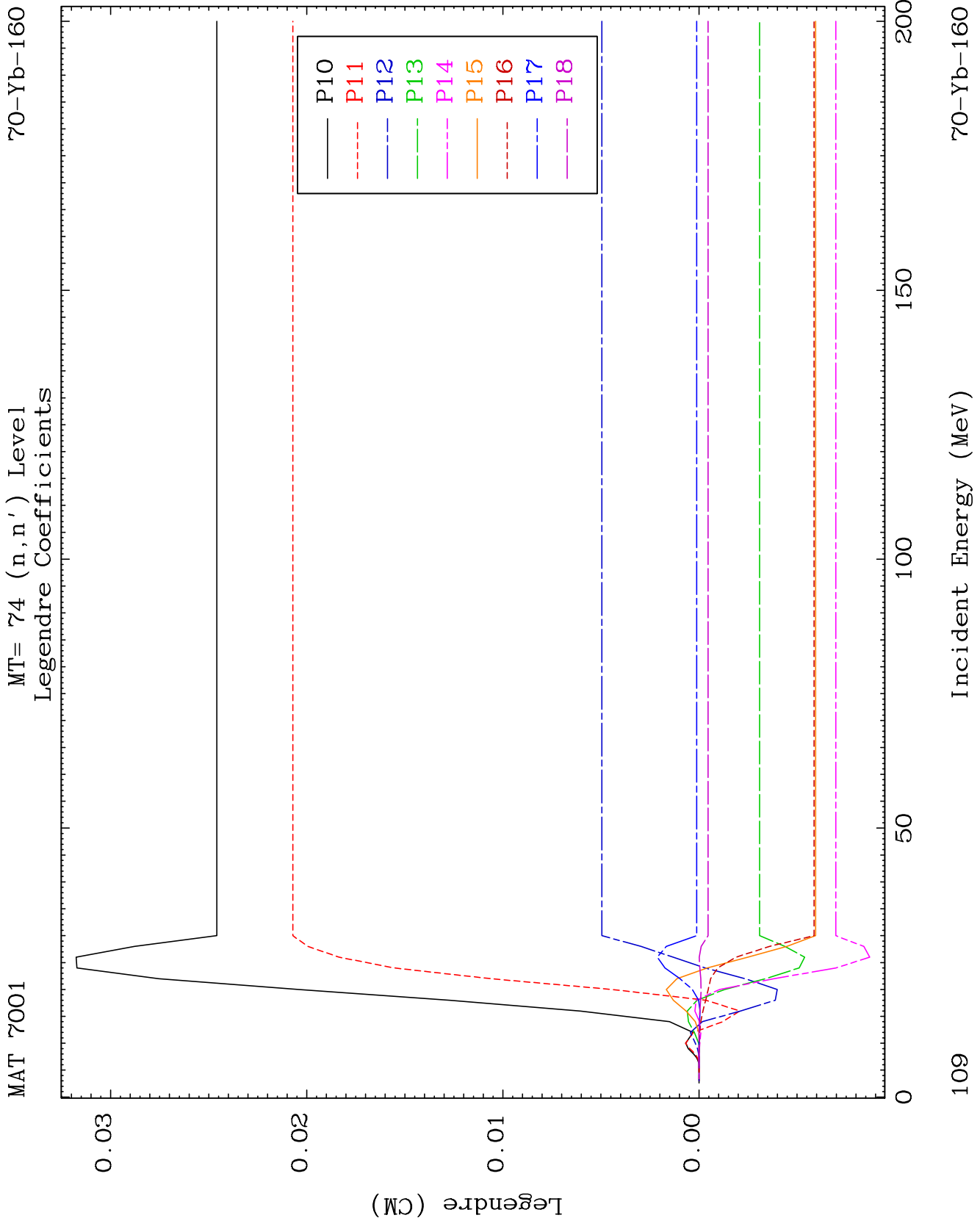


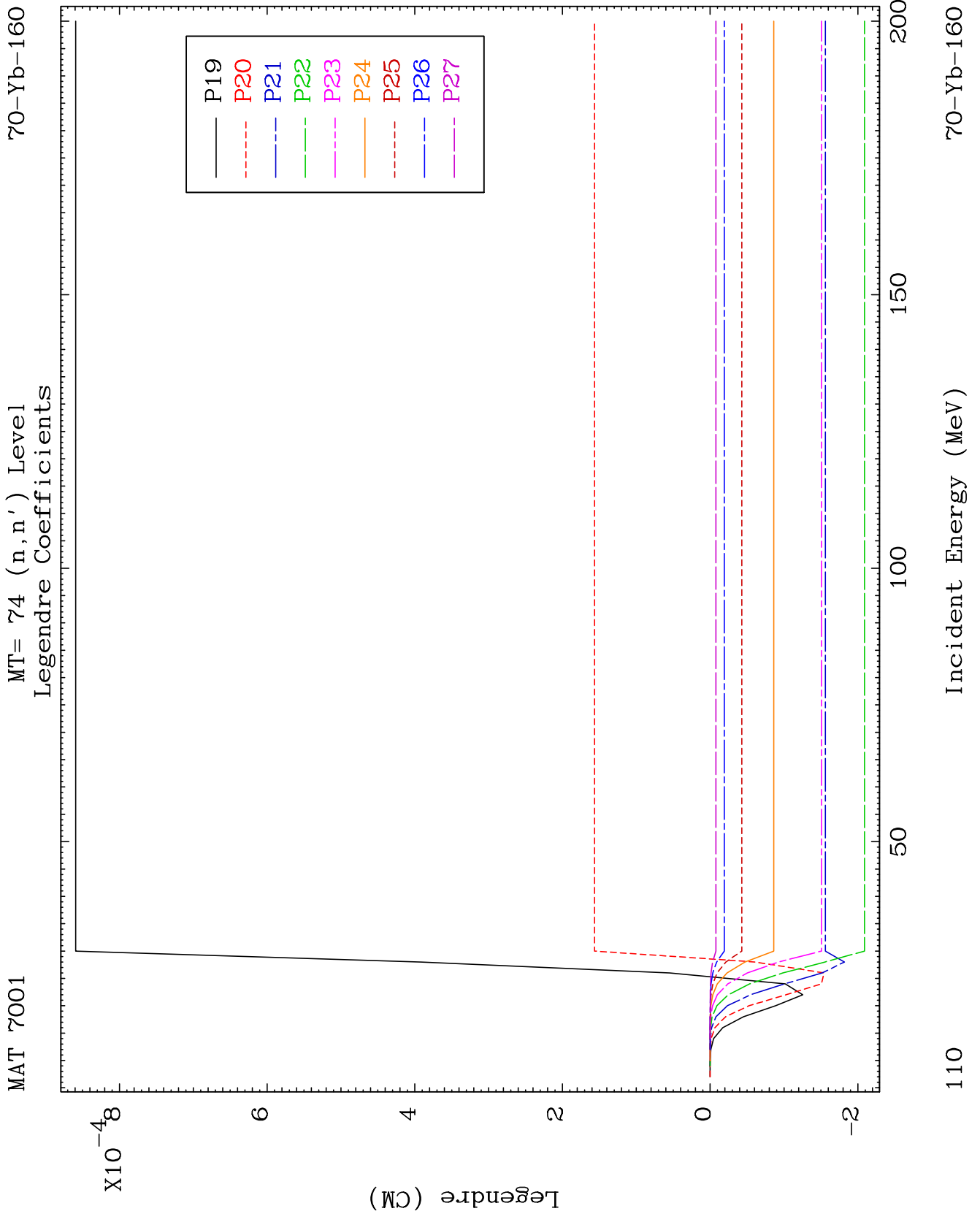


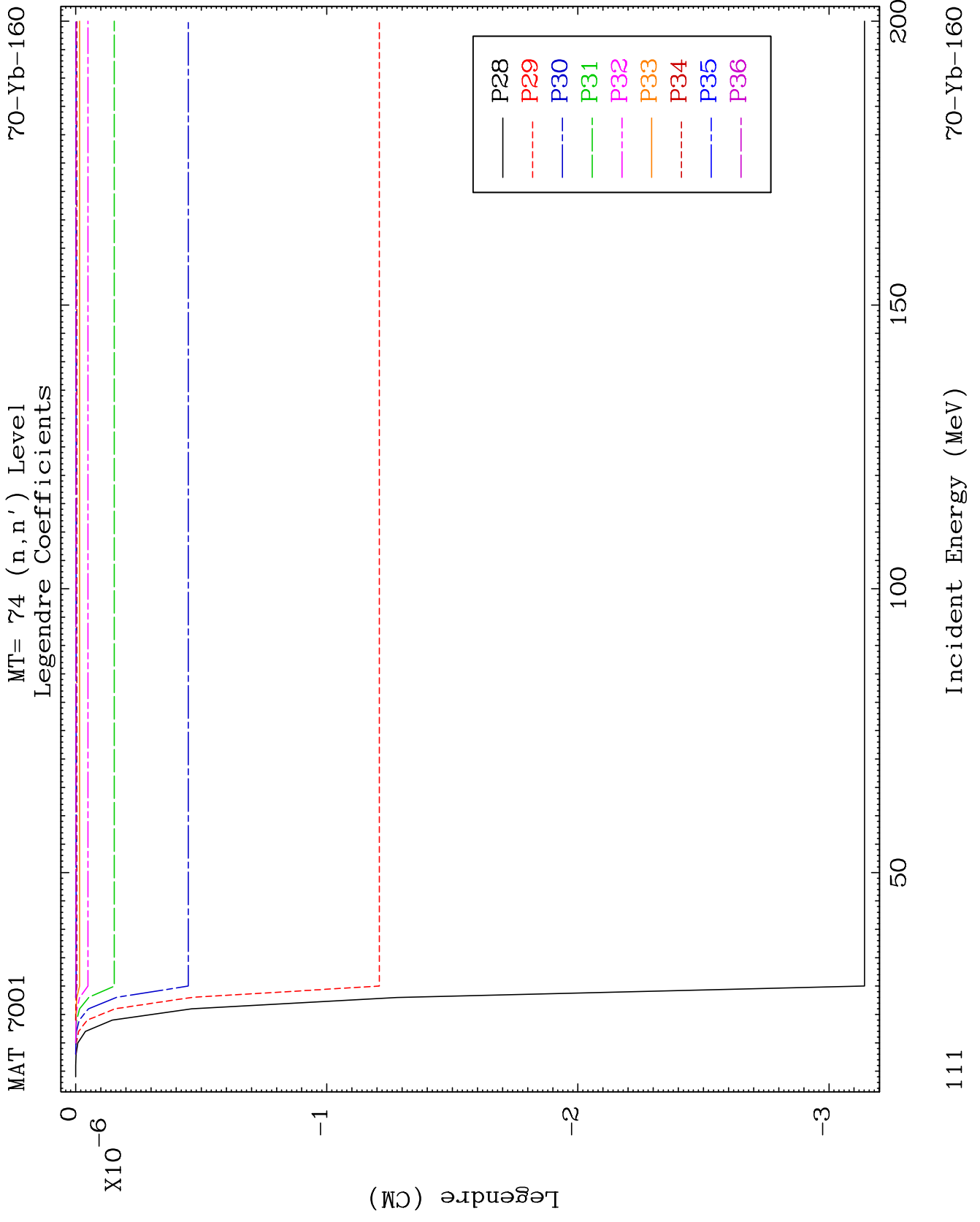


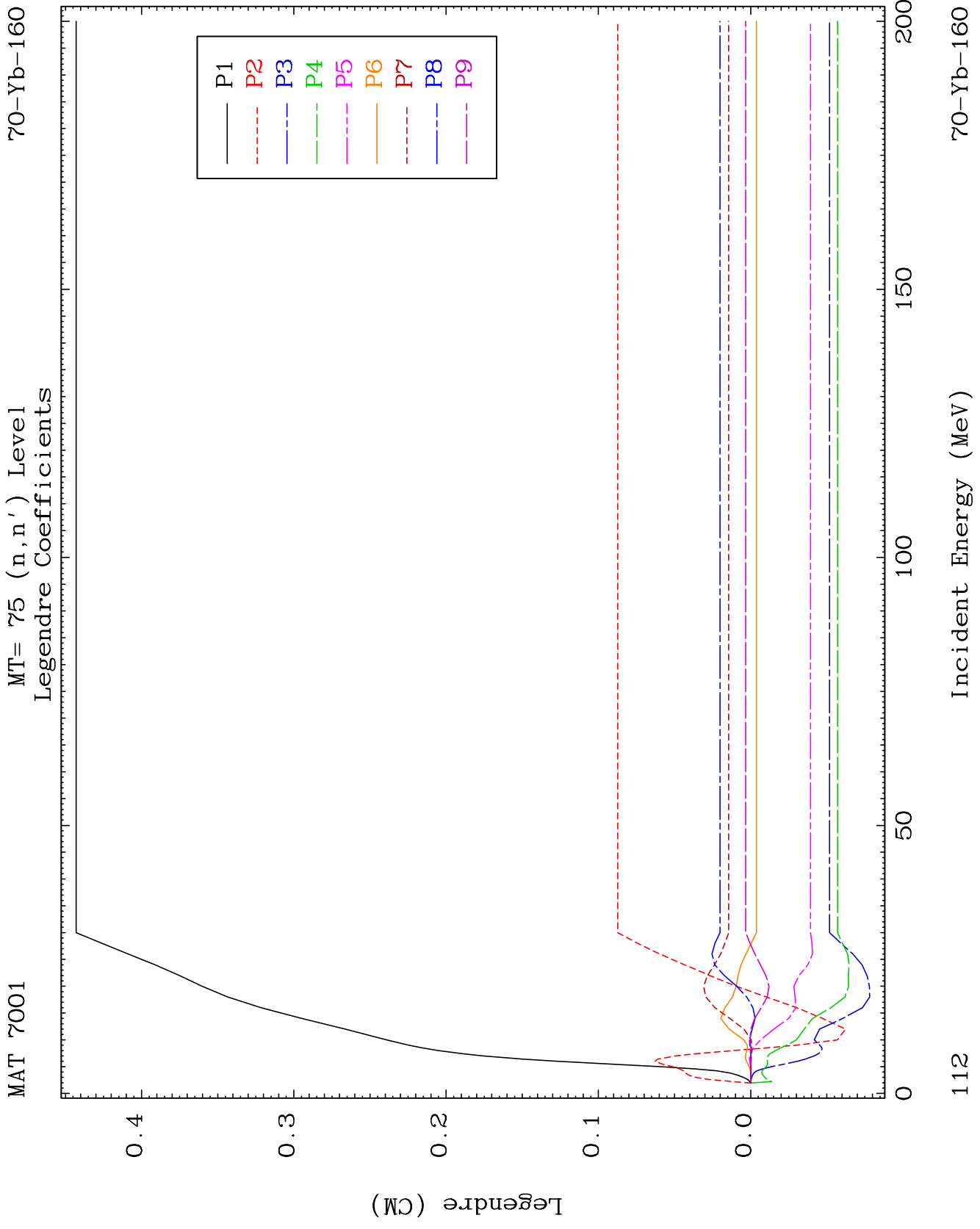


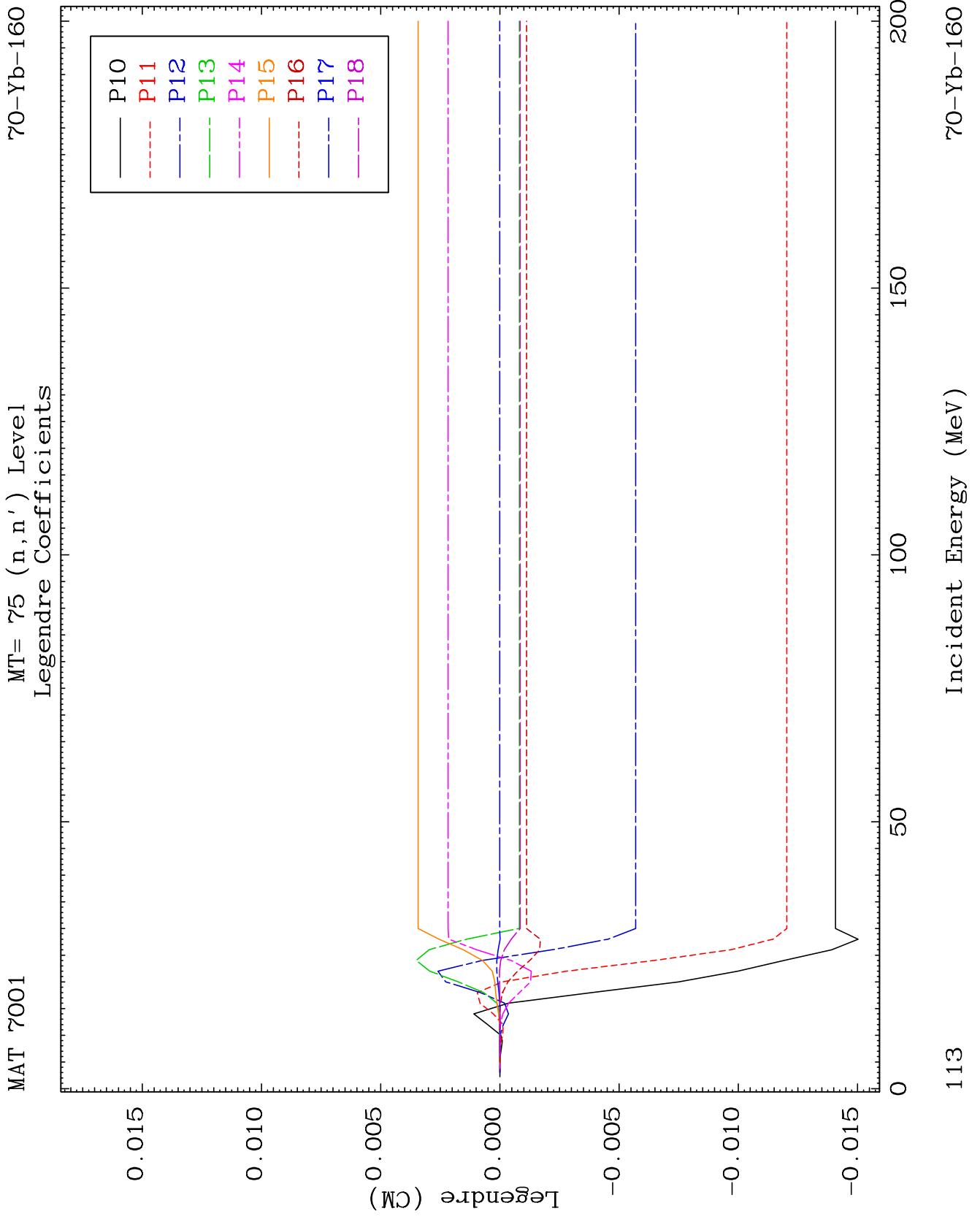


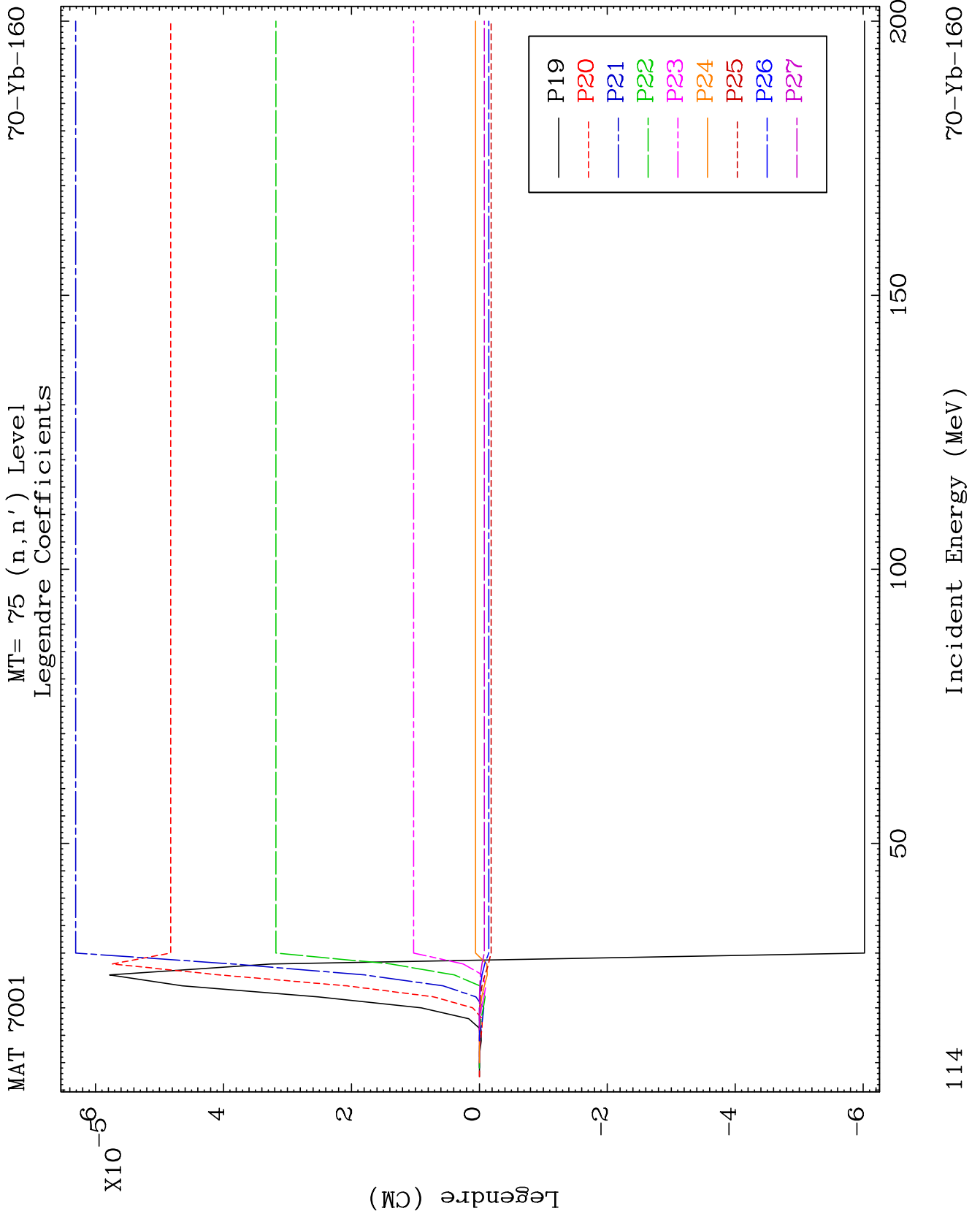


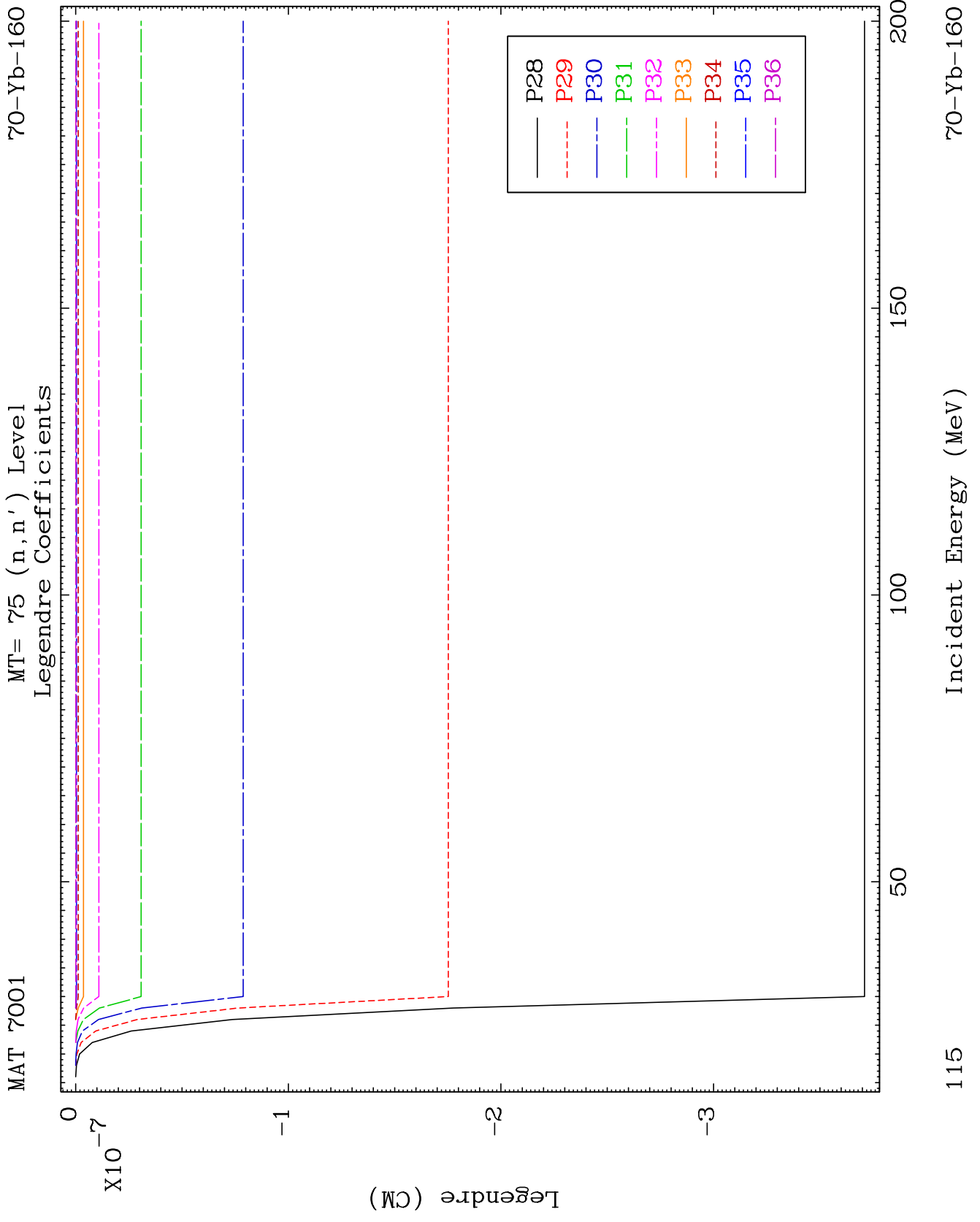


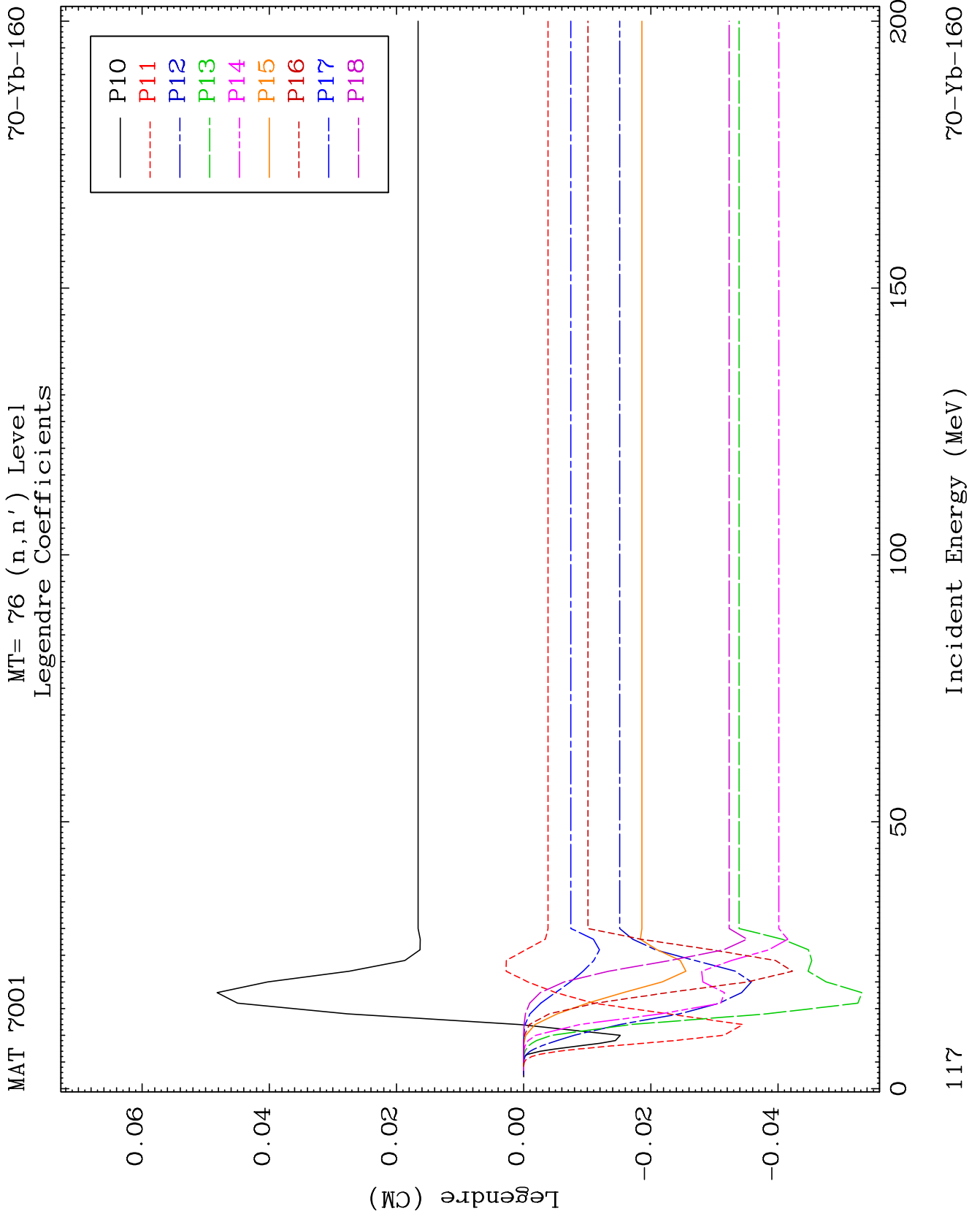


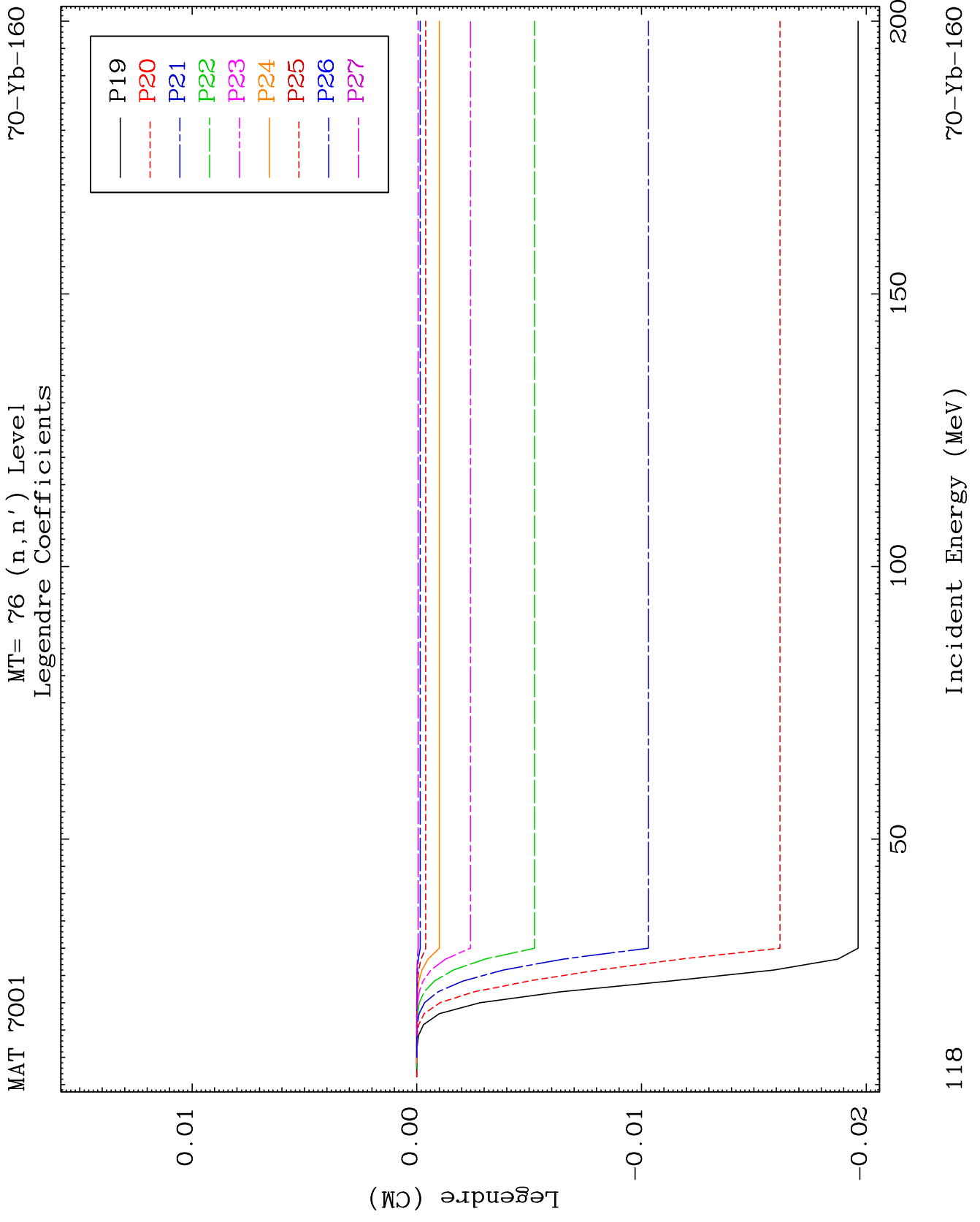


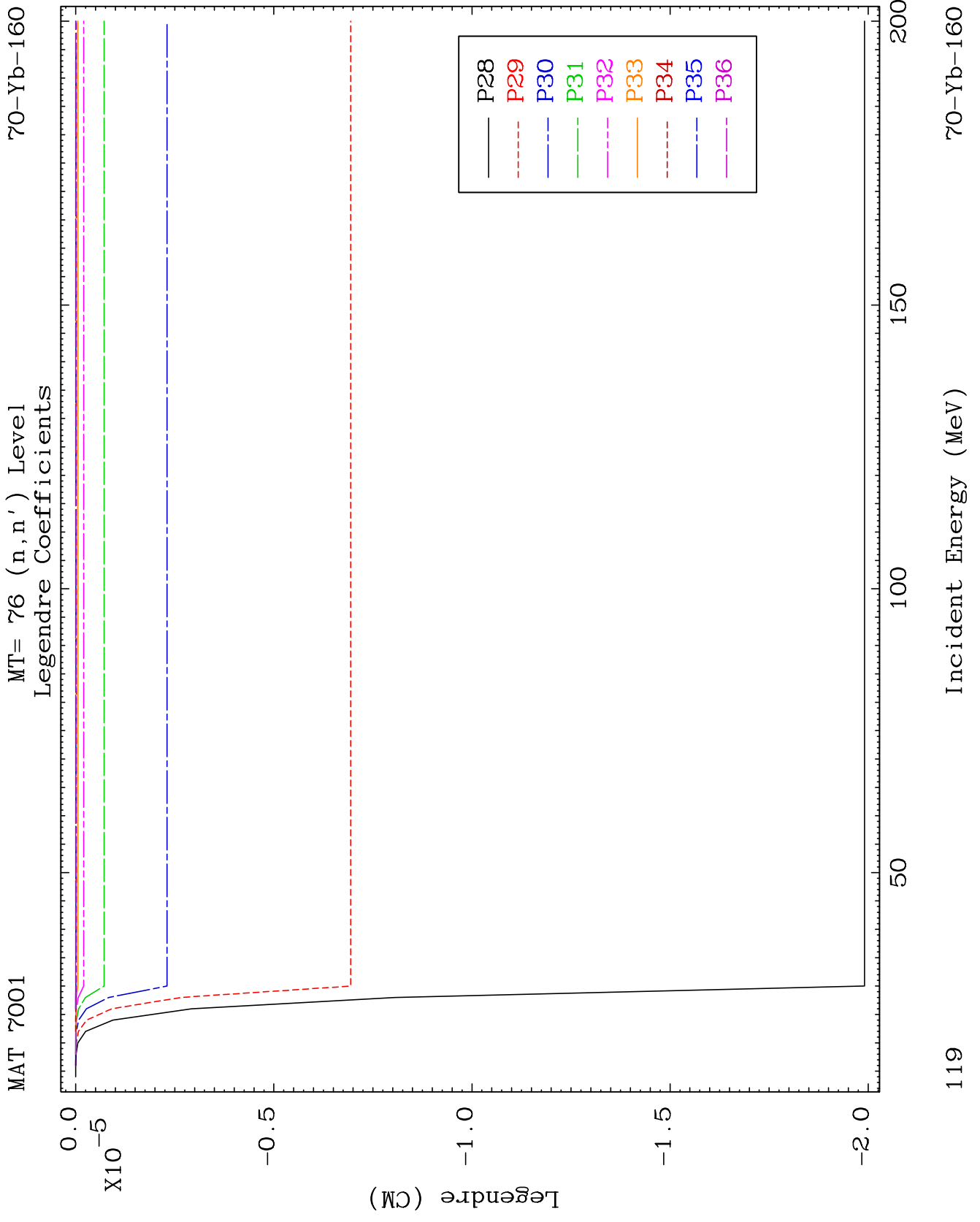










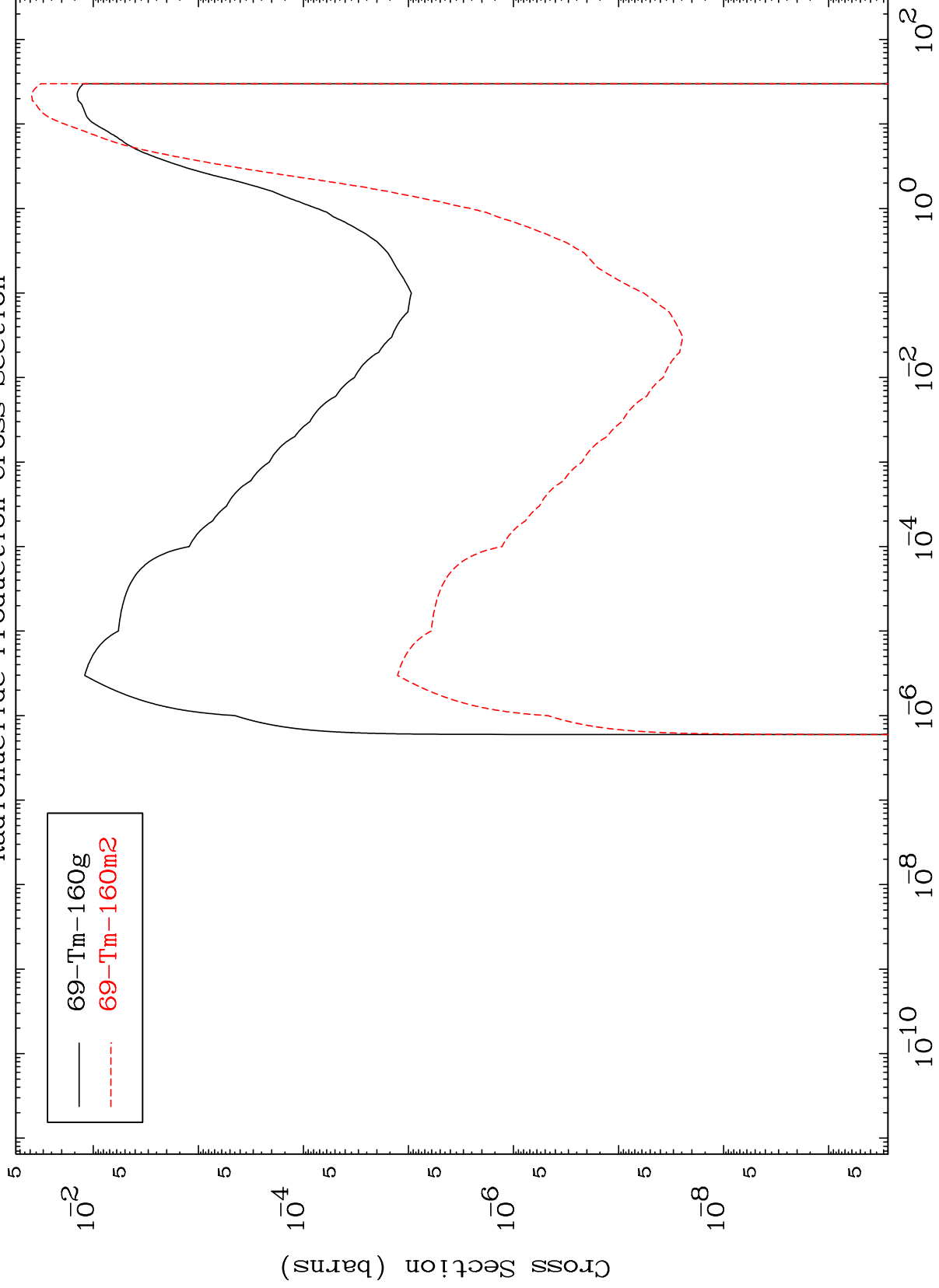


MAT 7001

(n,p)

70-Yb-160

Radionuclide Production Cross Section



— 69-Tm-160g
- - - 69-Tm-160m2

120

Incident Energy (MeV)

70-Yb-160

MAT 7001

(n,p) α

70-Yb-160

Radionuclide Production Cross Section

