

Program EVALPLOT
(Version 2021-1)

by

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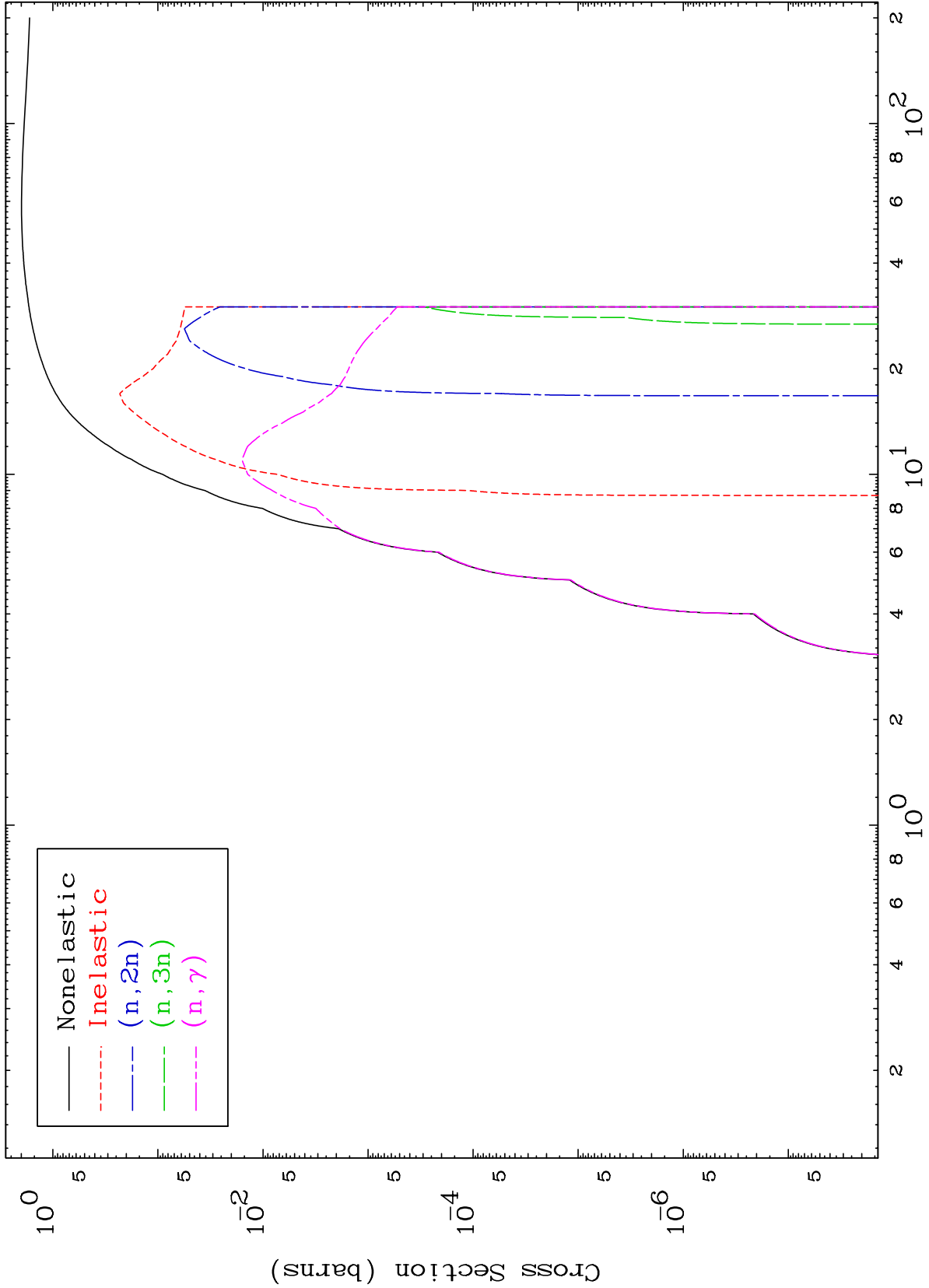
Press Mouse Button to Start

MAT 8610

Proton Major

86-Rn-206

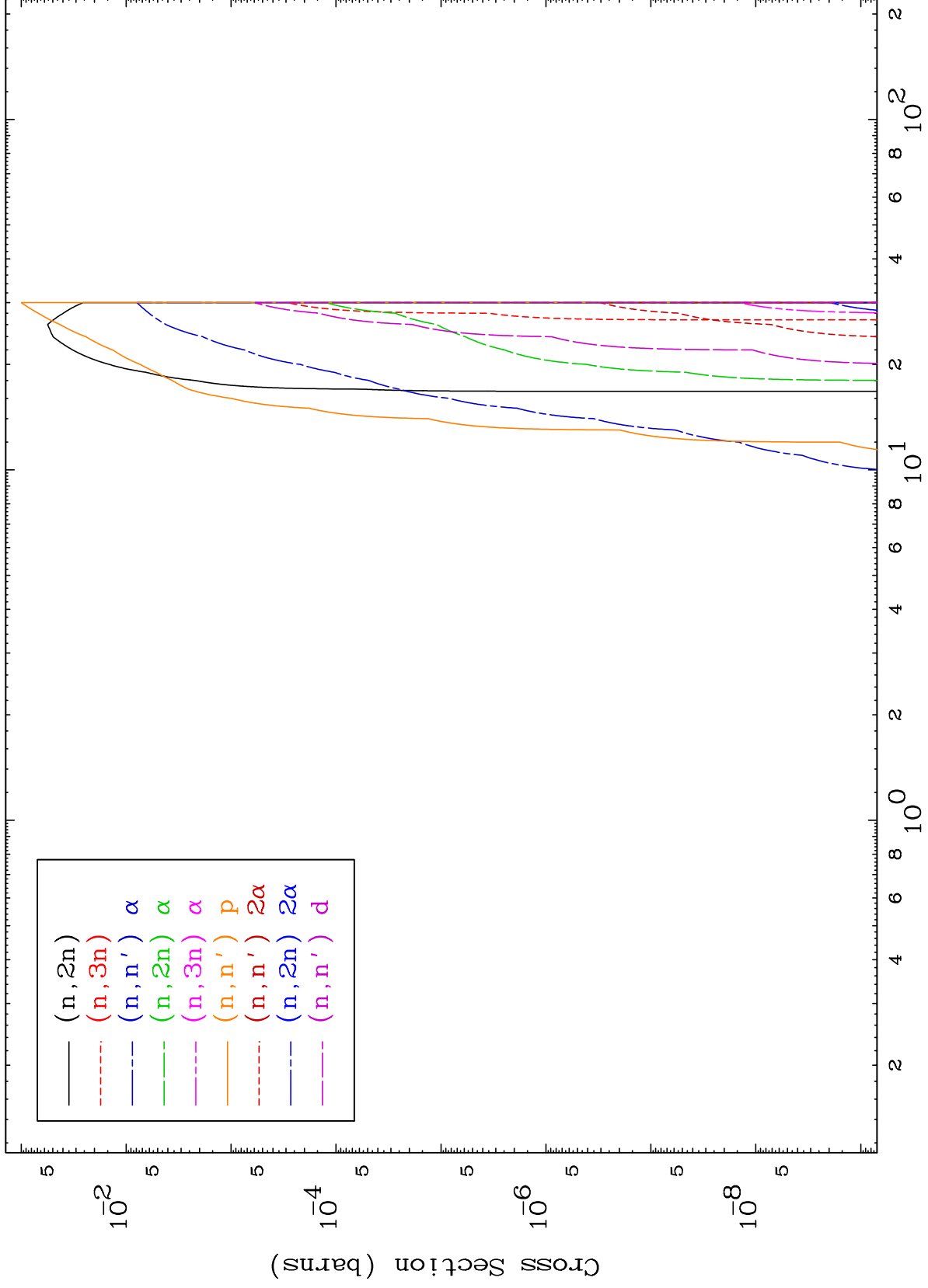
0 Kelvin Cross Sections

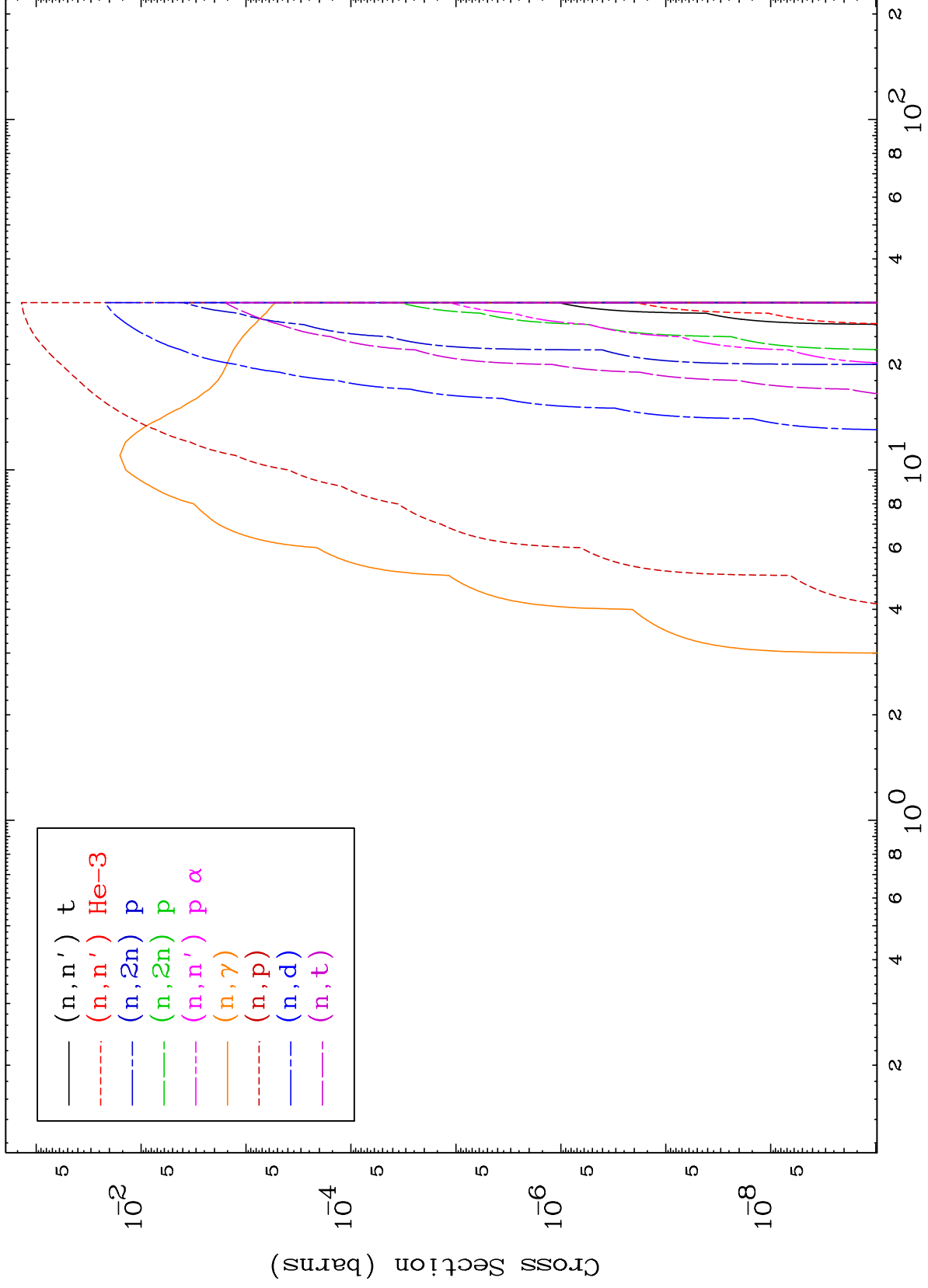


MAT 8610

Proton Neutron Absorption
0 Kelvin Cross Sections

86-Rn-206

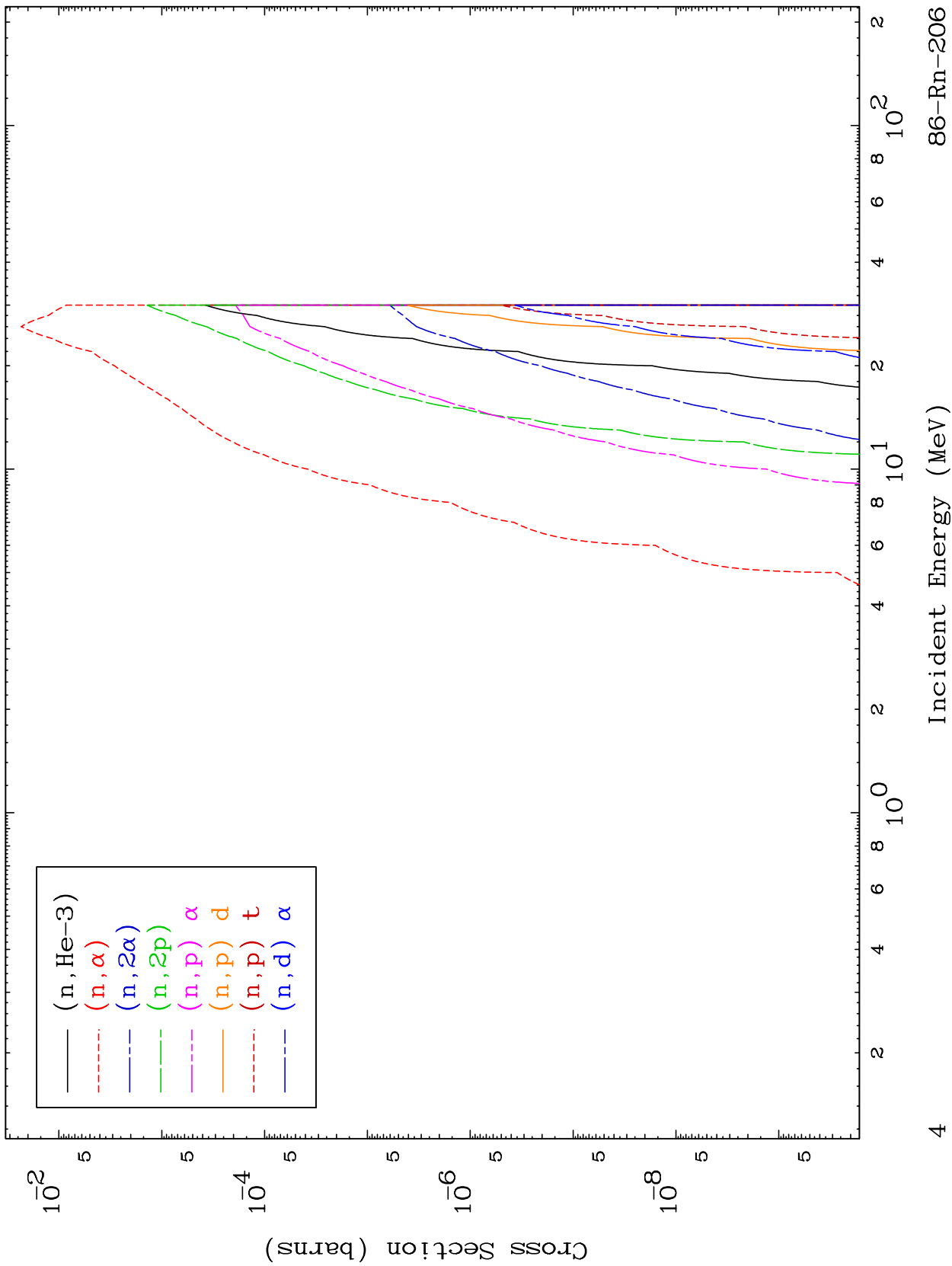




MAT 8610

Proton Neutron Absorption
0 Kelvin Cross Sections

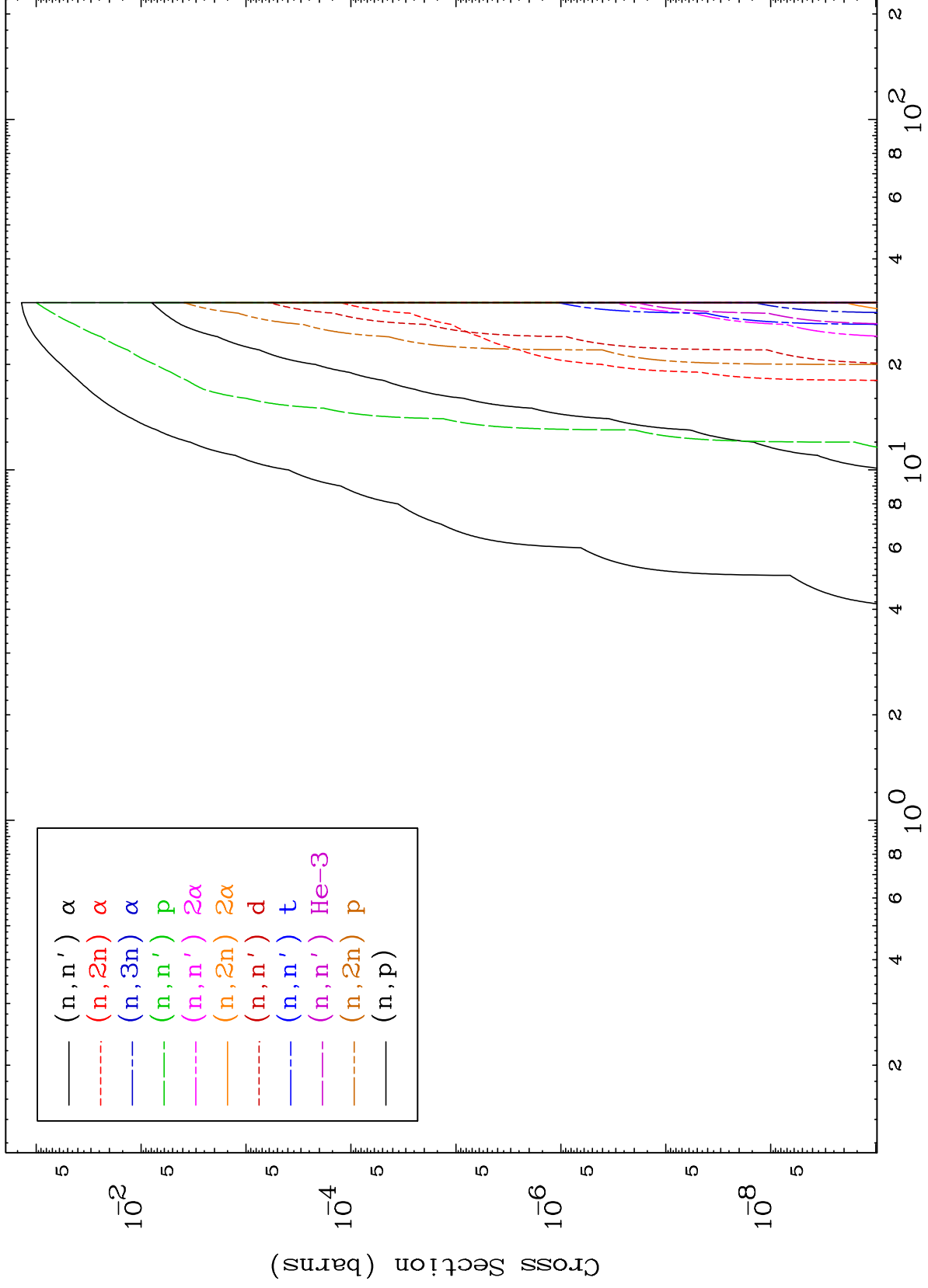
86-Rn-206



MAT 8610

Proton Charged Particle
0 Kelvin Cross Sections

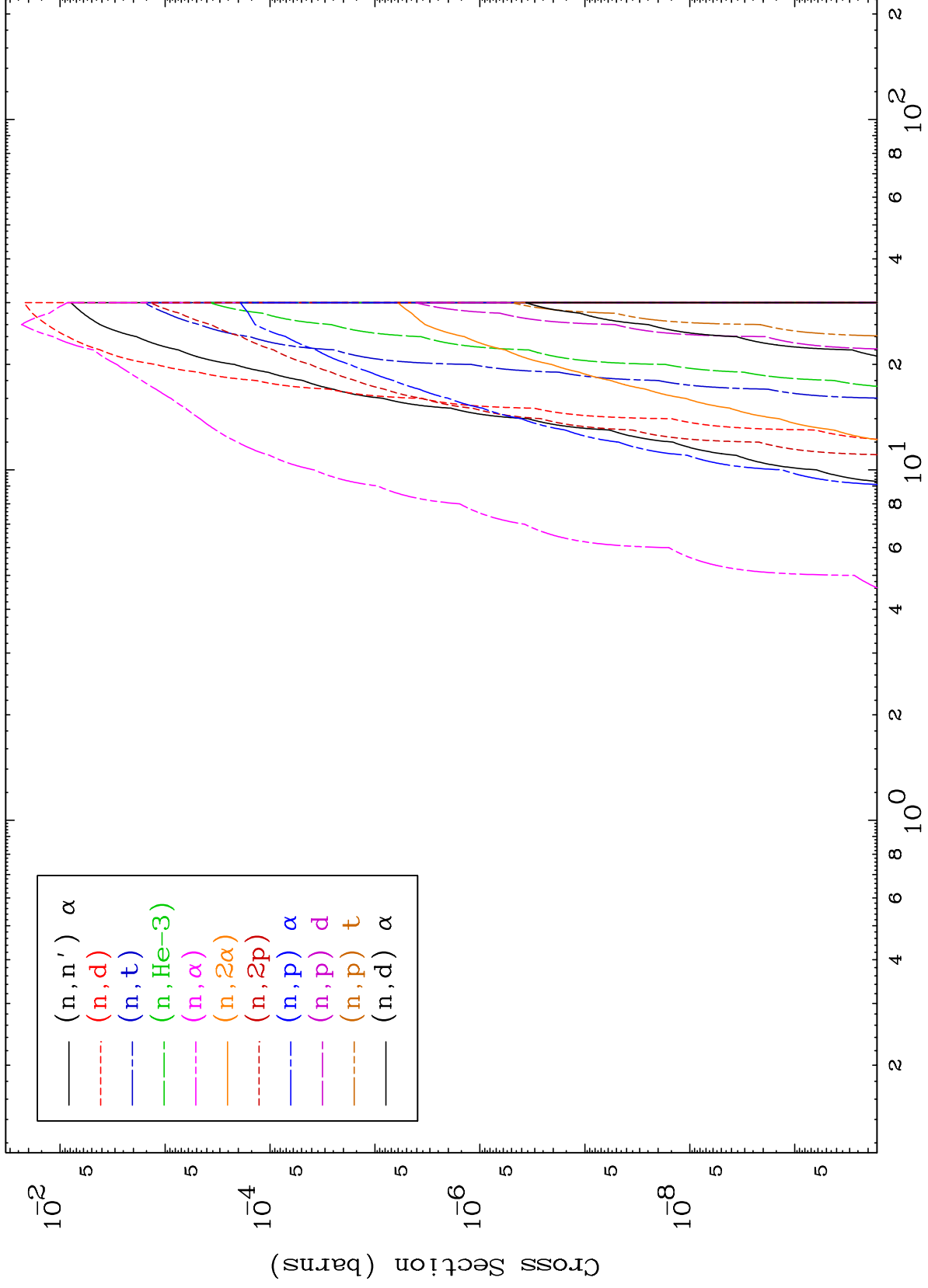
86-Rn-206



MAT 8610

Proton Charged Particle
0 Kelvin Cross Sections

86-Rn-206

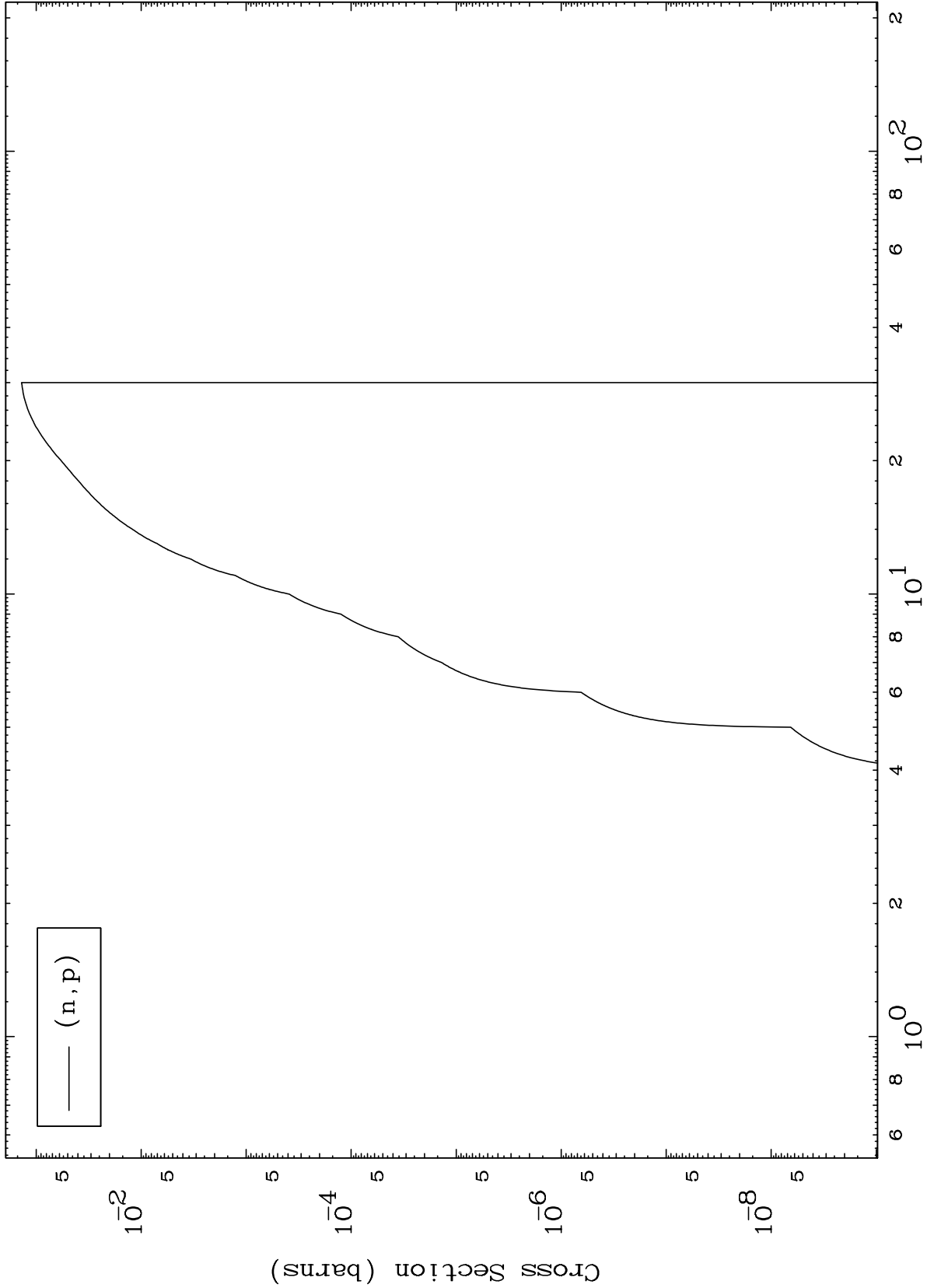


MAT 8610

(p,p) Levels

86-Rn-206

0 Kelvin Cross Sections



(n,p)

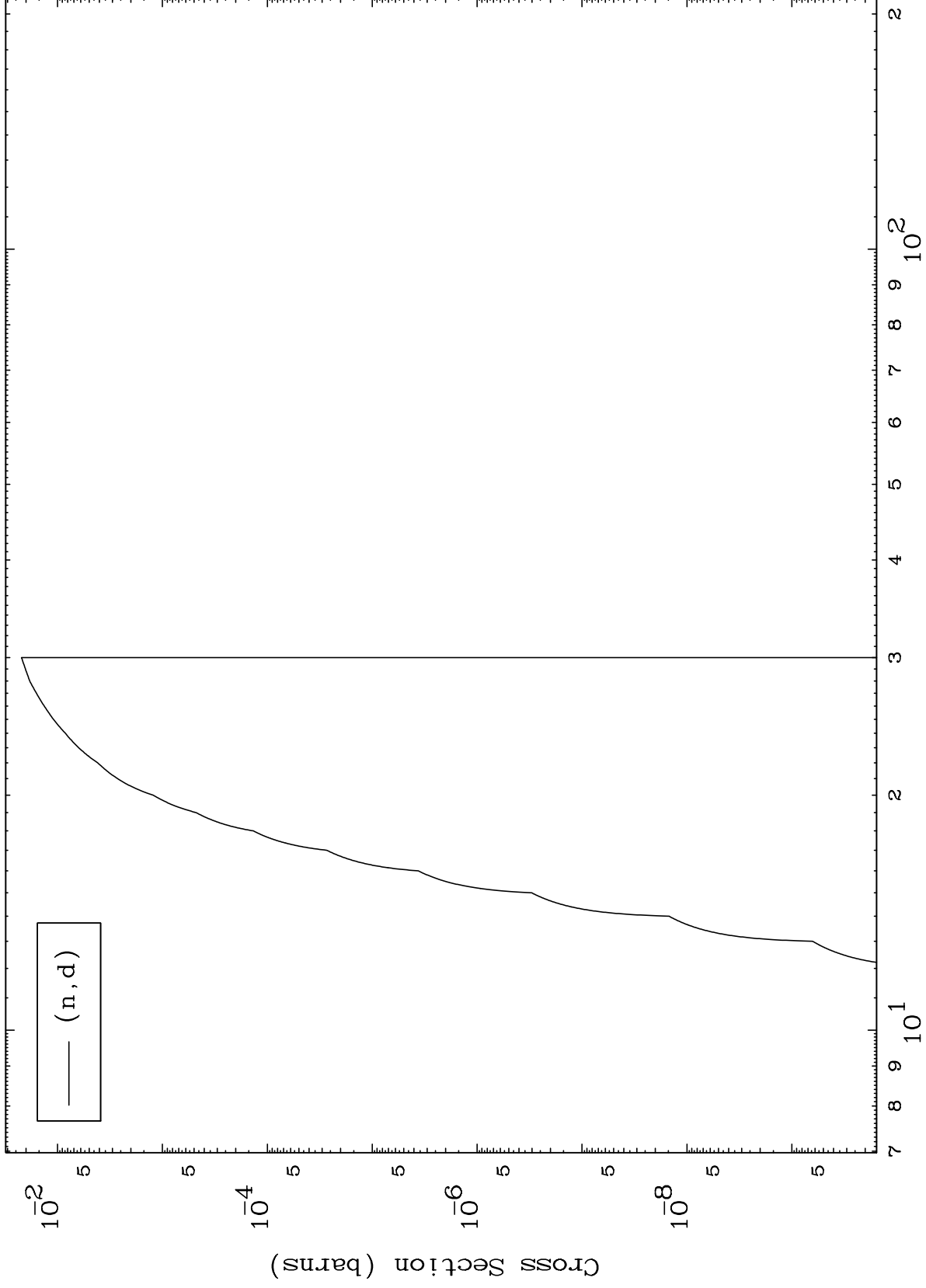
Incident Energy (MeV)

86-Rn-206

MAT 8610

(p,d) Levels
0 Kelvin Cross Sections

86-Rn-206



8

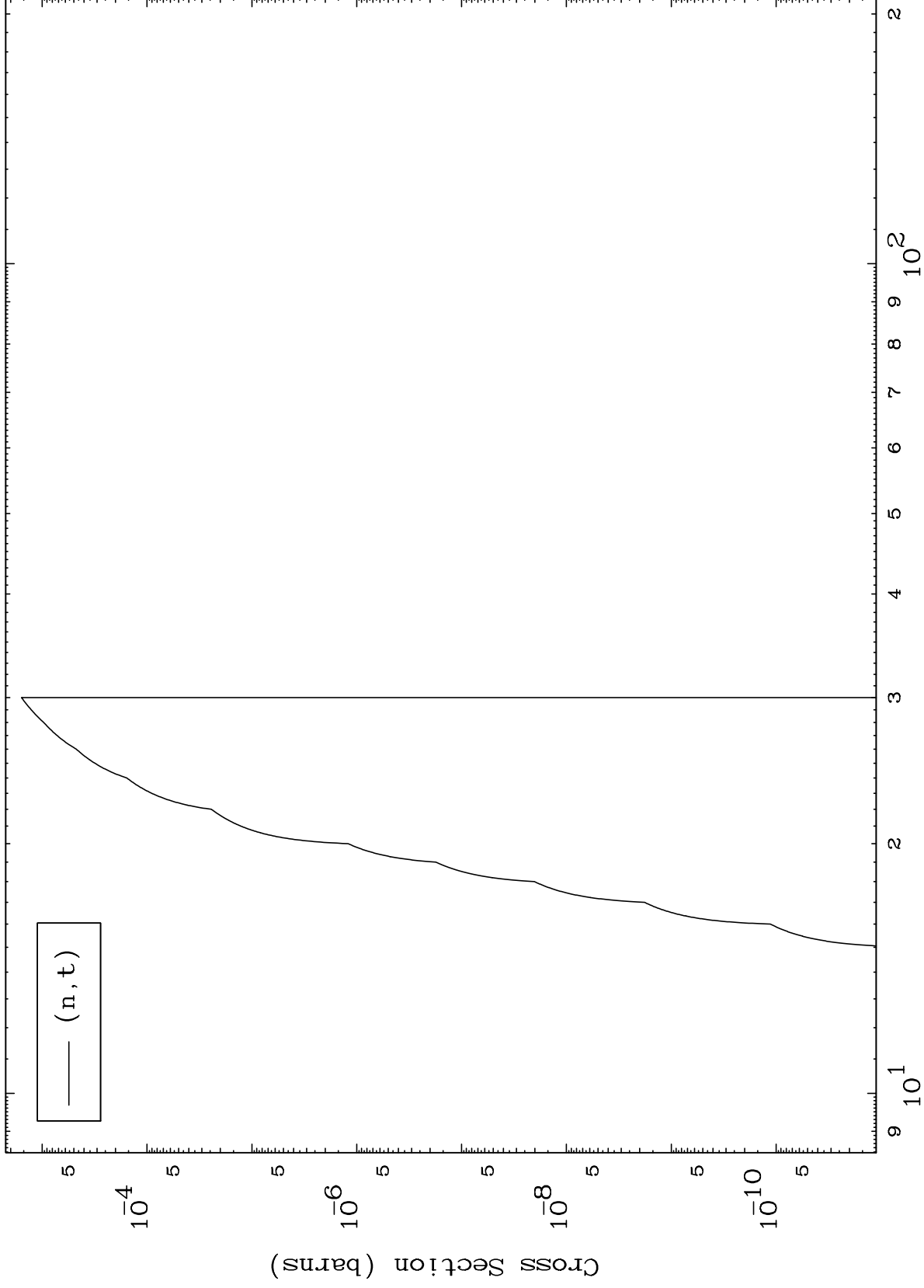
Incident Energy (MeV)

86-Rn-206

MAT 8610

(p,t) Levels
0 Kelvin Cross Sections

86-Rn-206



9

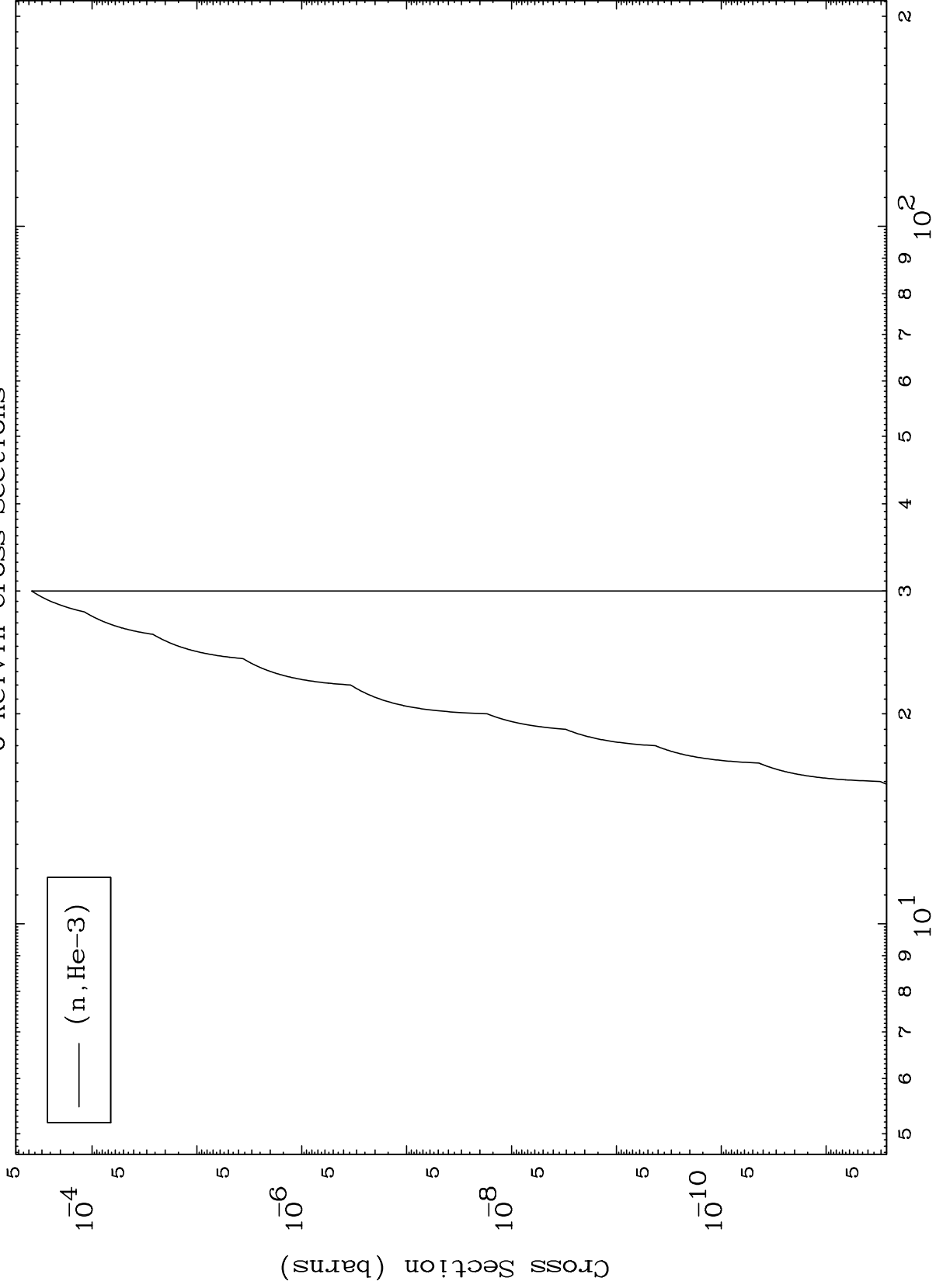
Incident Energy (MeV)

86-Rn-206

MAT 8610

(p,He3) Levels
0 Kelvin Cross Sections

86-Rn-206



10

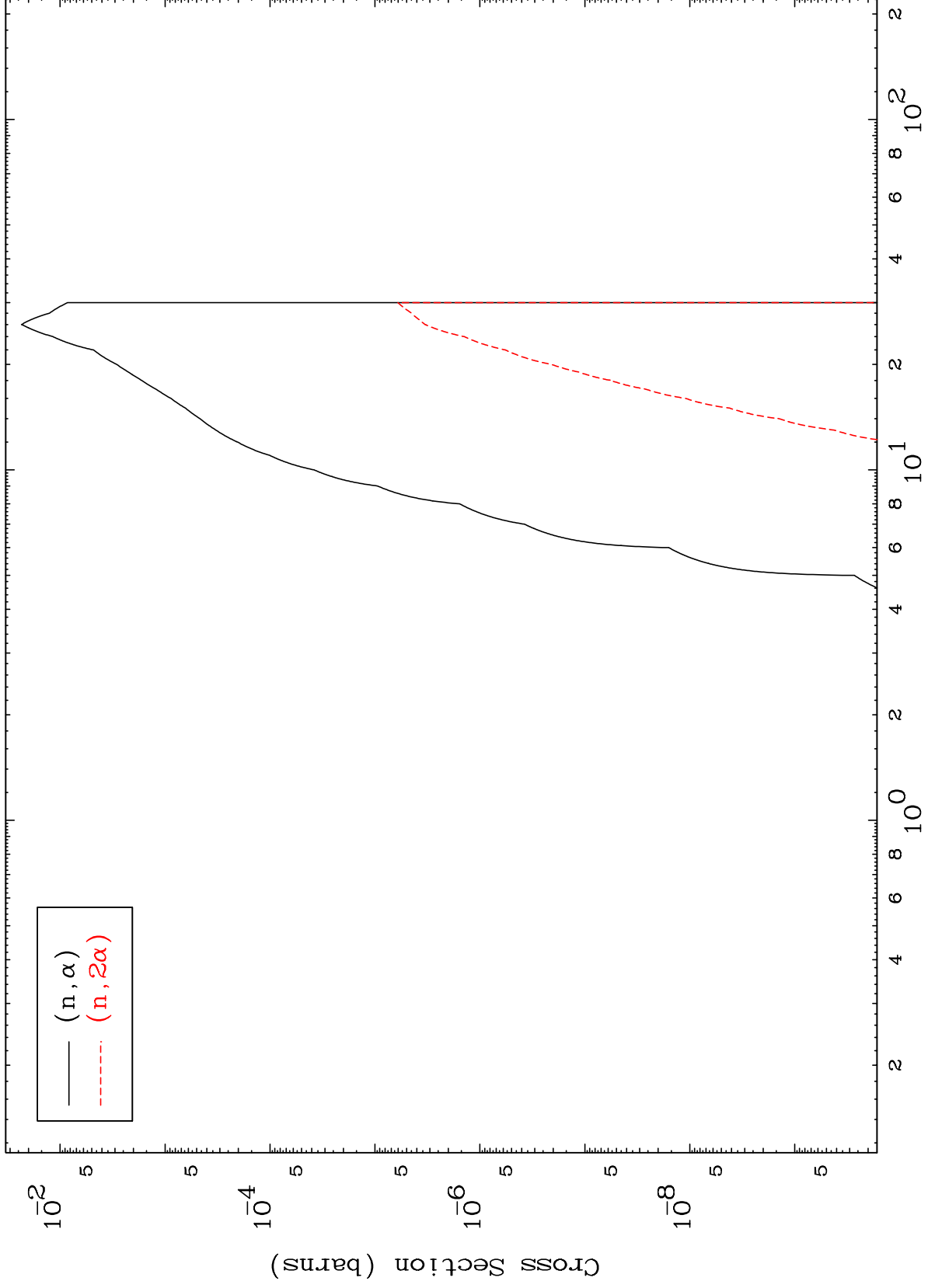
Incident Energy (MeV)

86-Rn-206

MAT 8610

(p, α) Levels
0 Kelvin Cross Sections

86-Rn-206



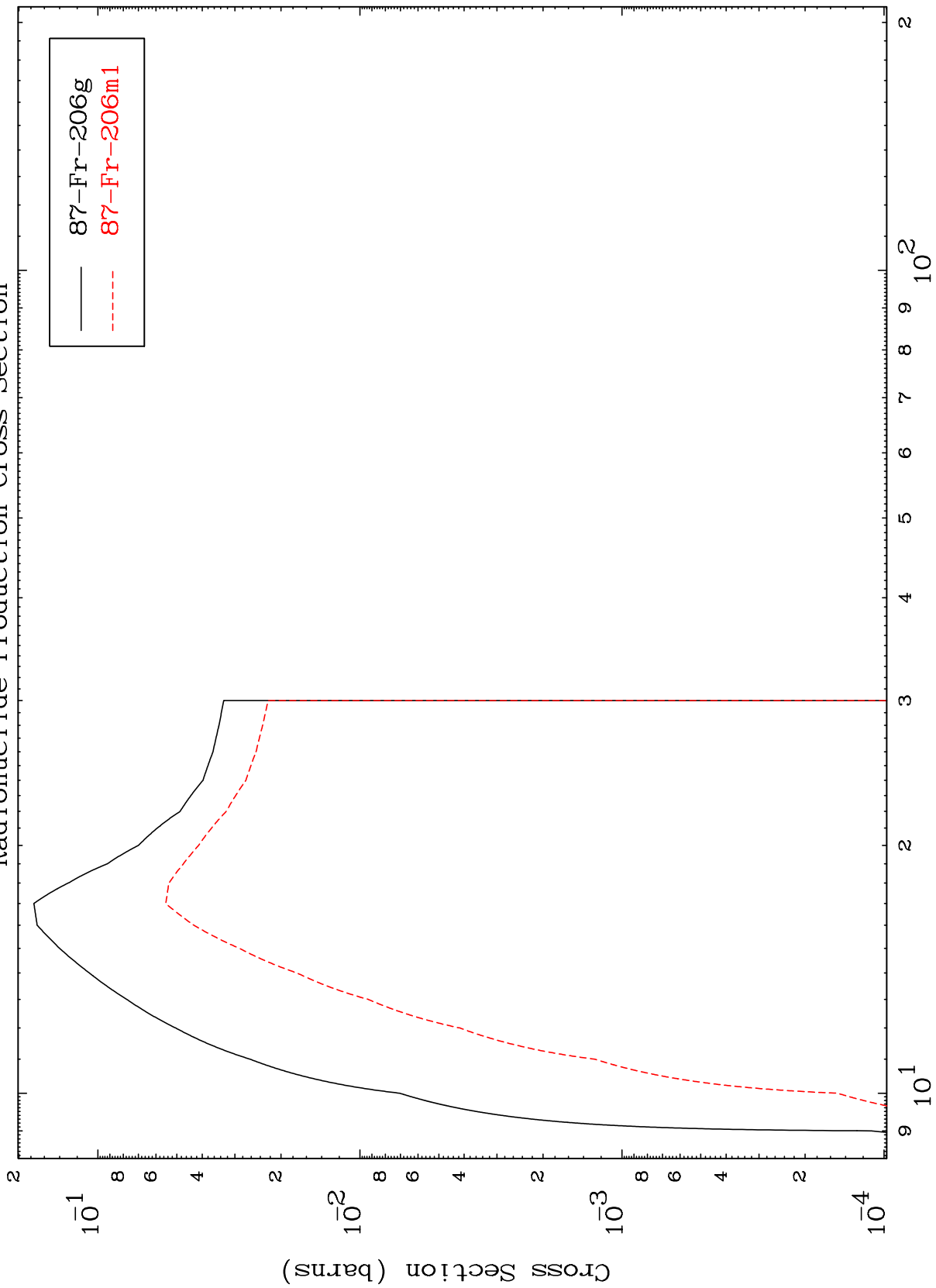
86-Rn-206

Incident Energy (MeV)

MAT 8610

86-Rn-206

Inelastic
Radionuclide Production Cross Section



86-Rn-206

Incident Energy (MeV)

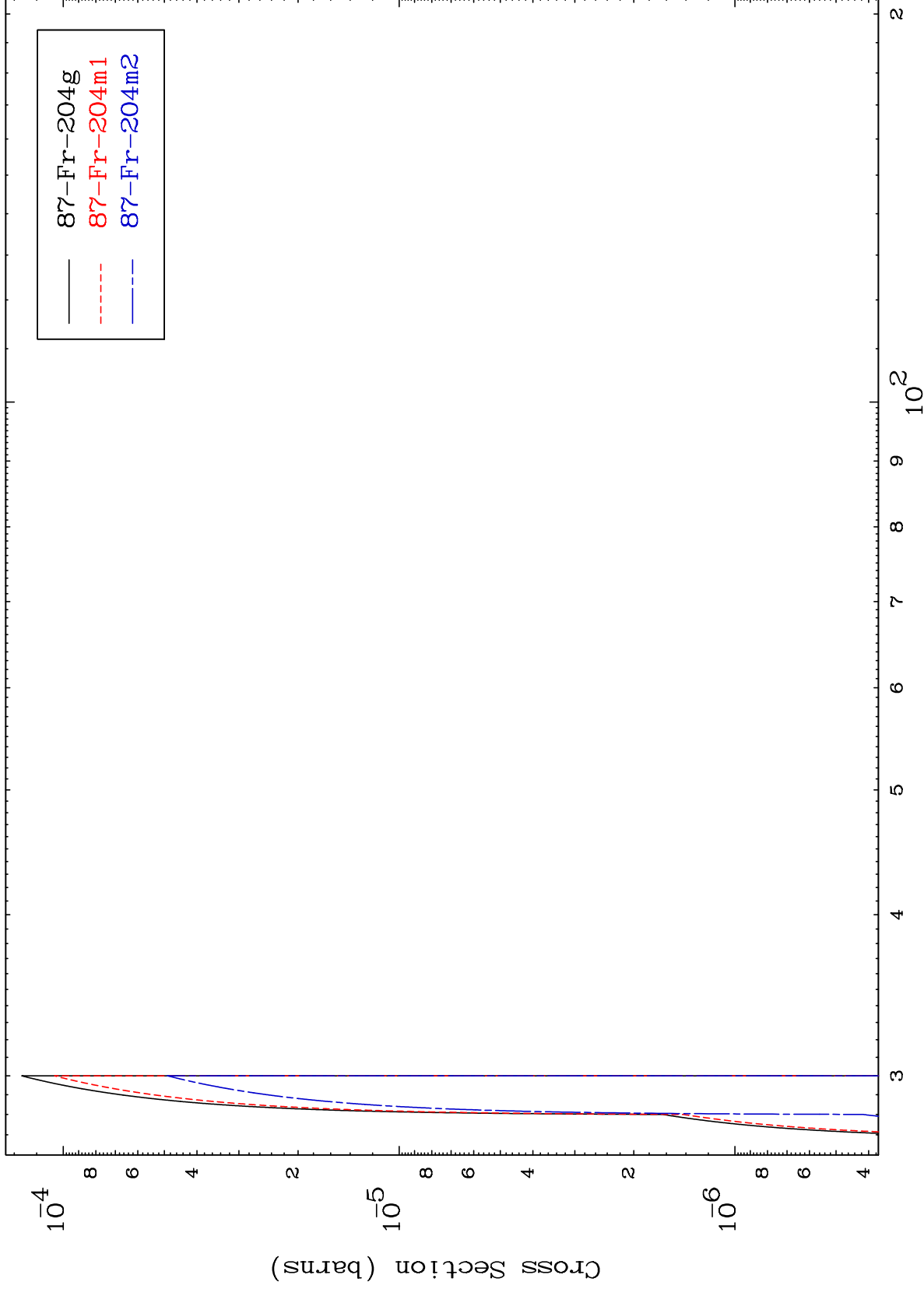
12

MAT 8610

(n,3n)

86-Rn-206

Radionuclide Production Cross Section



13

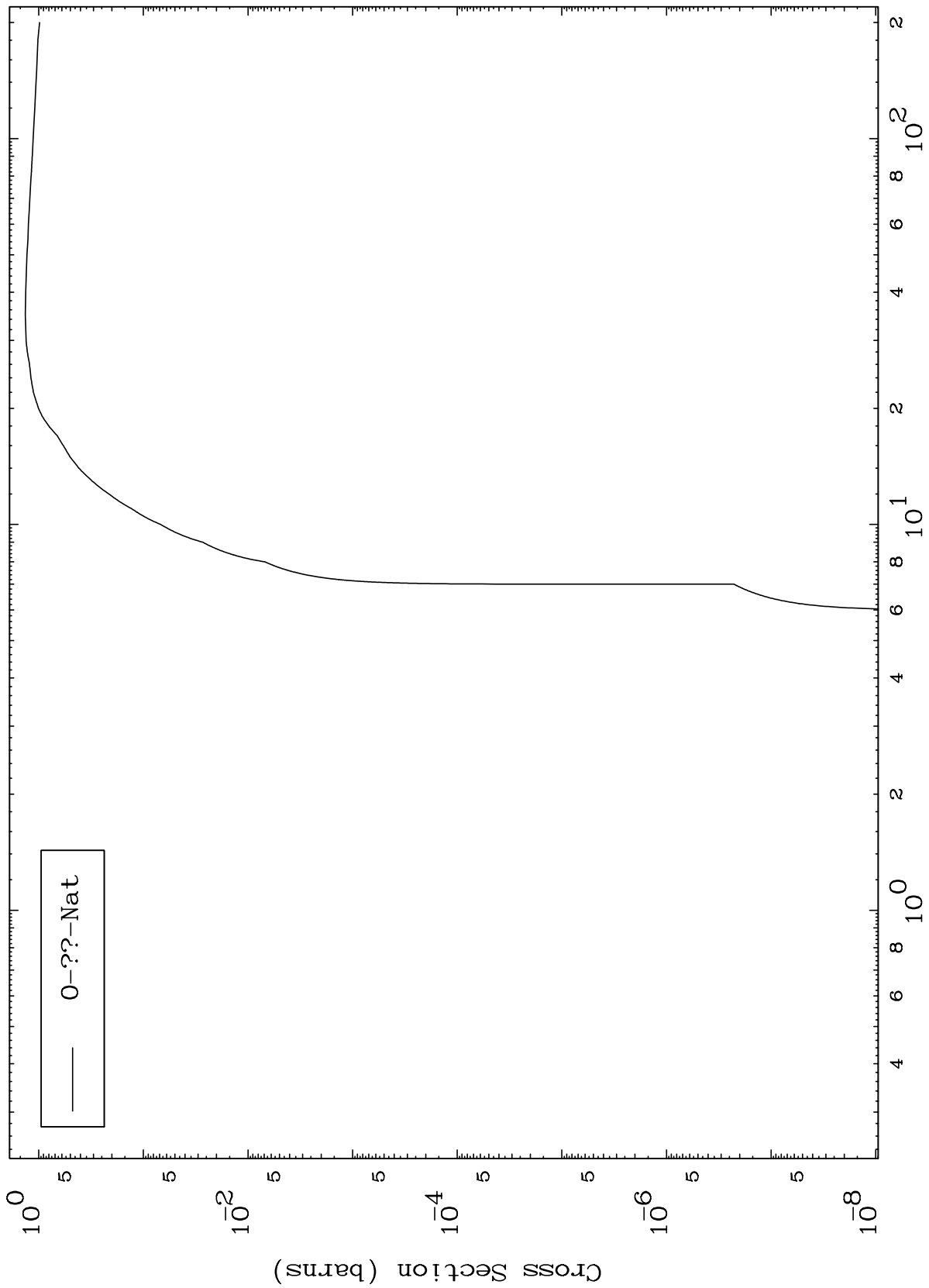
Incident Energy (MeV)

86-Rn-206

MAT 8610

86-Rn-206

Fission
Radionuclide Production Cross Section



14

86-Rn-206

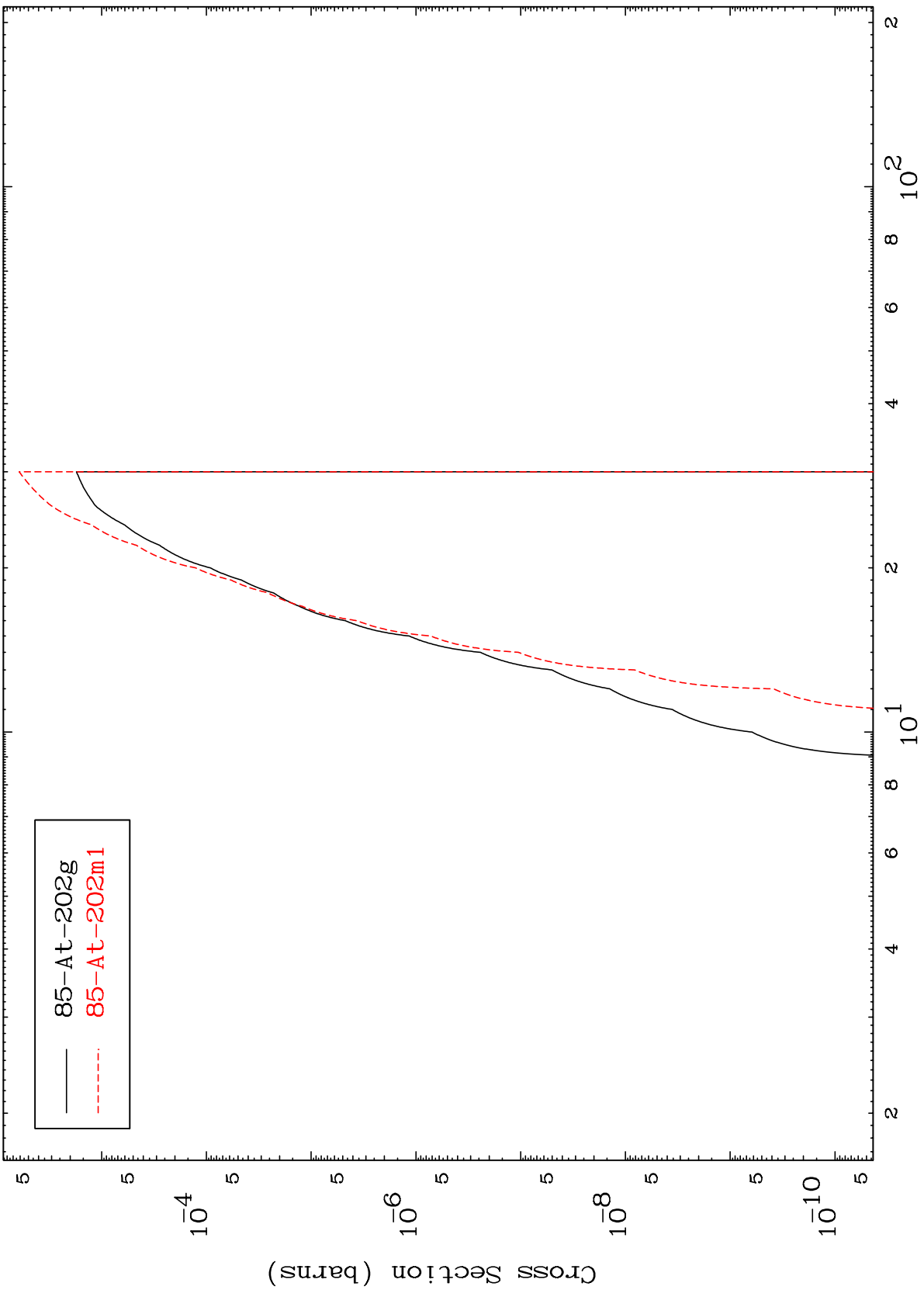
Incident Energy (MeV)

MAT 8610

$(n, n') \alpha$

86-Rn-206

Radionuclide Production Cross Section



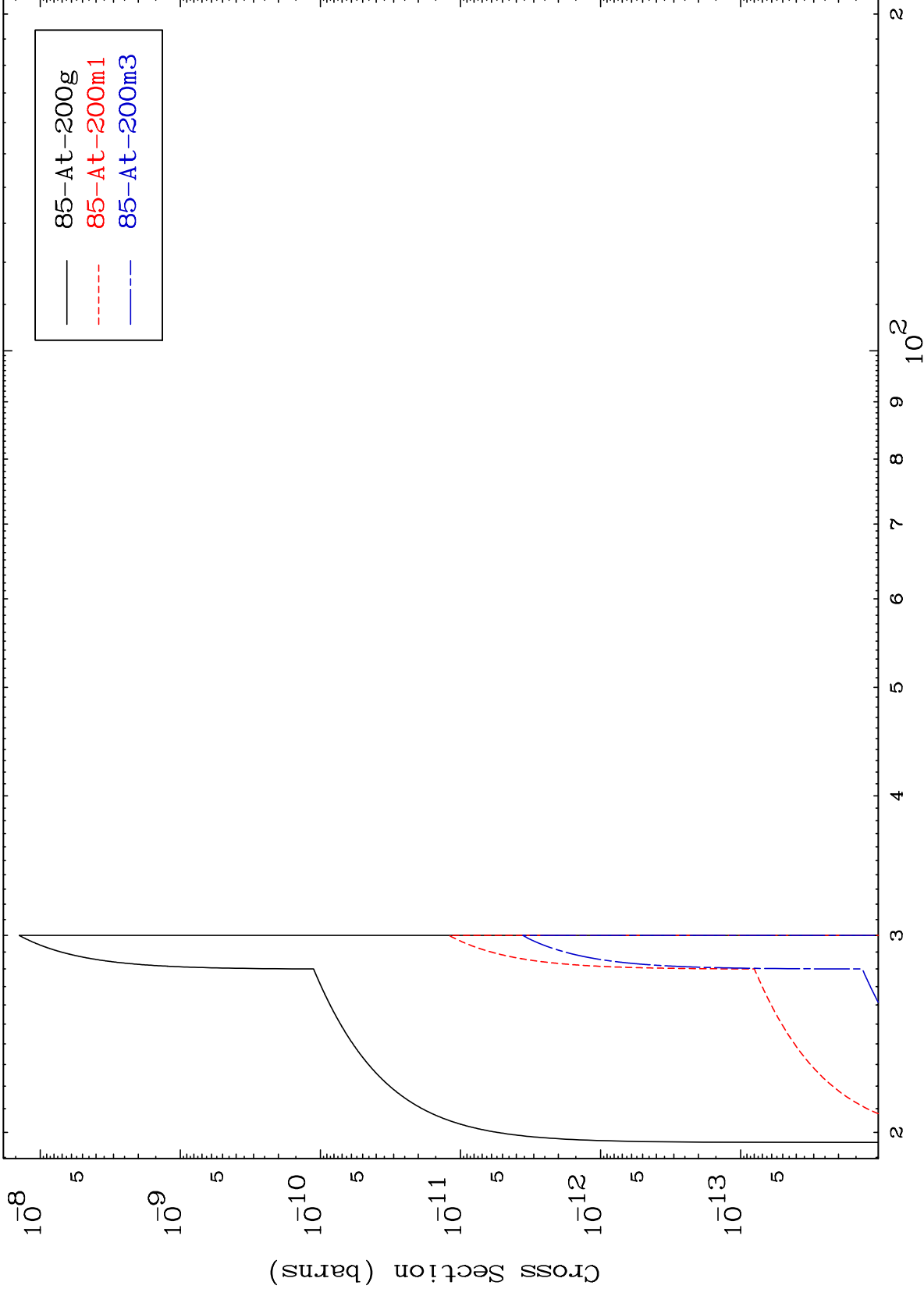
85-At-202g
85-At-202m1

15

Incident Energy (MeV)

86-Rn-206

Radionuclide Production Cross Section

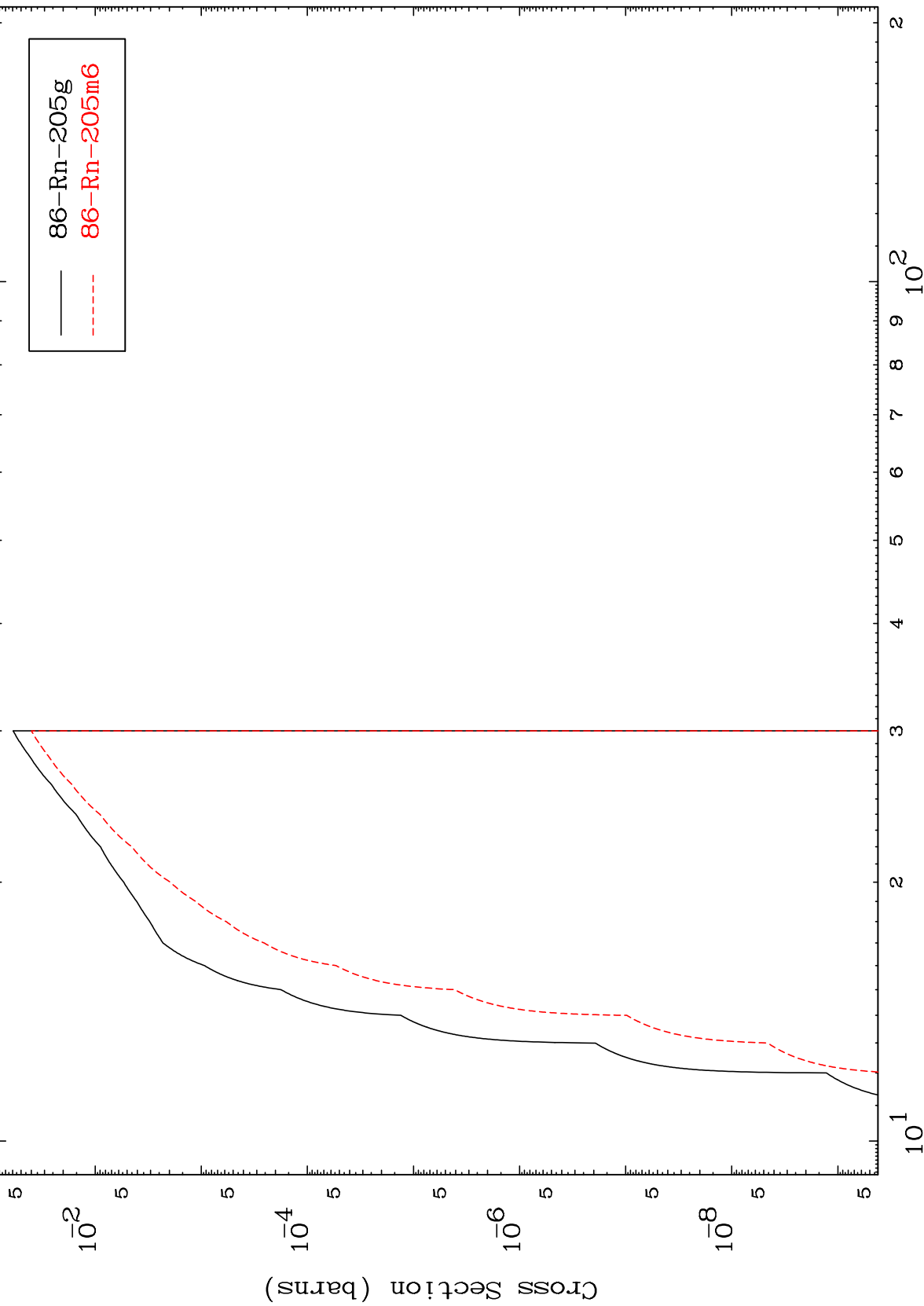


MAT 8610

(n,n') p

86-Rn-206

Radionuclide Production Cross Section



17

Incident Energy (MeV)

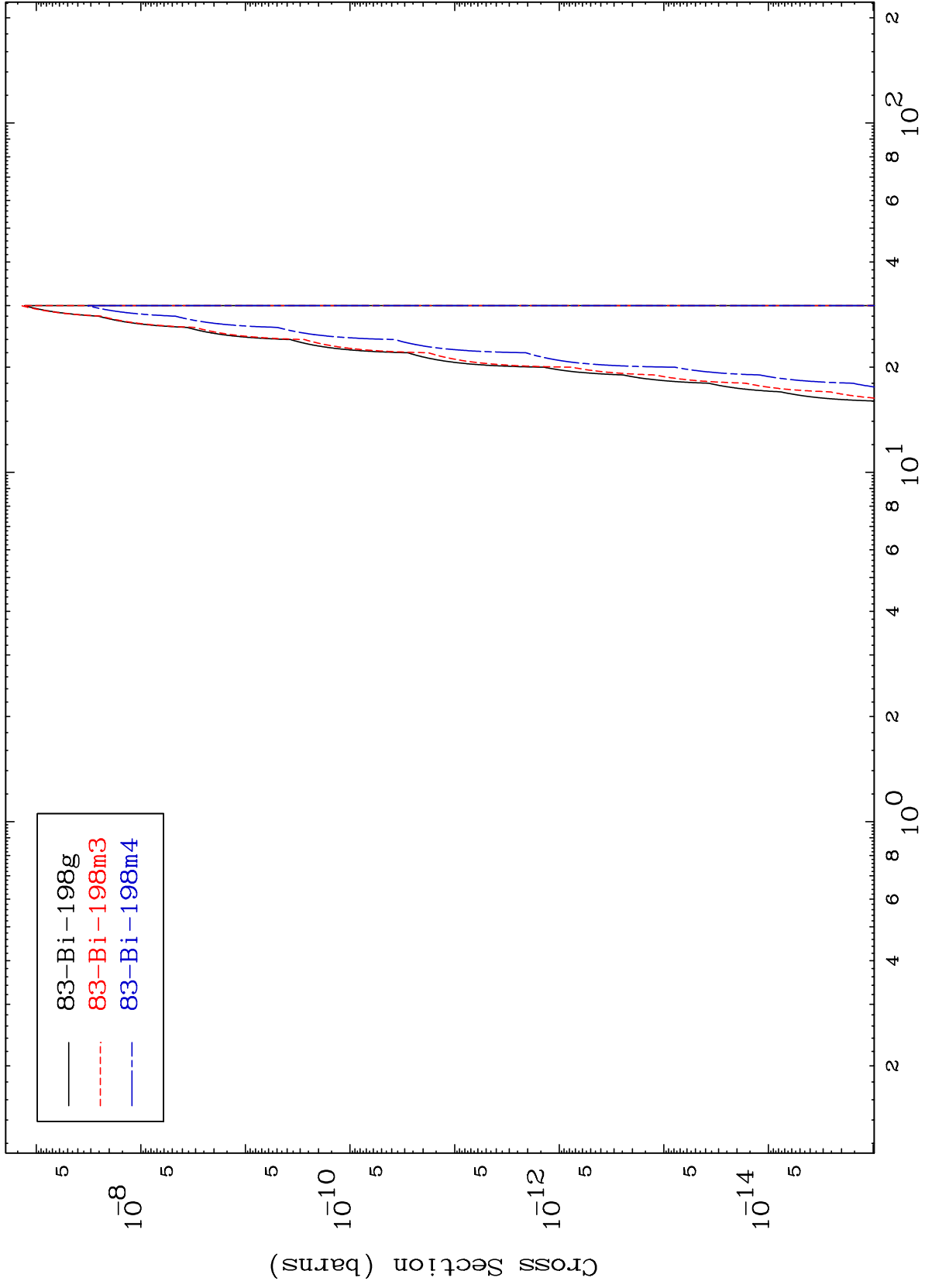
86-Rn-206

MAT 8610

(n,n') 2α

86-Rn-206

Radionuclide Production Cross Section



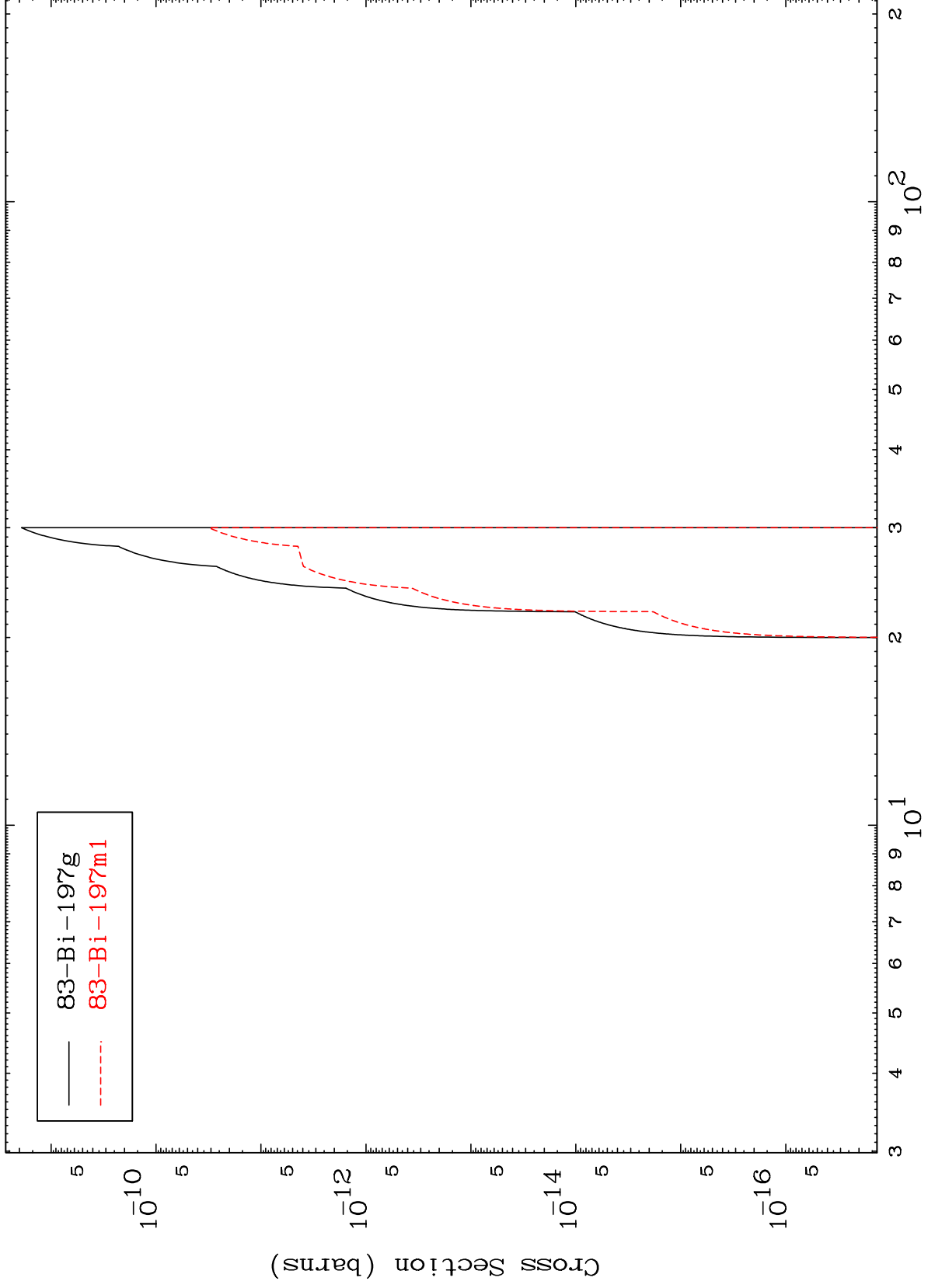
83-Bi-198g
83-Bi-198m3
83-Bi-198m4

MAT 8610

$^{86}\text{Rn-206}$ 2α

$^{86}\text{Rn-206}$

Radionuclide Production Cross Section



19

Incident Energy (MeV)

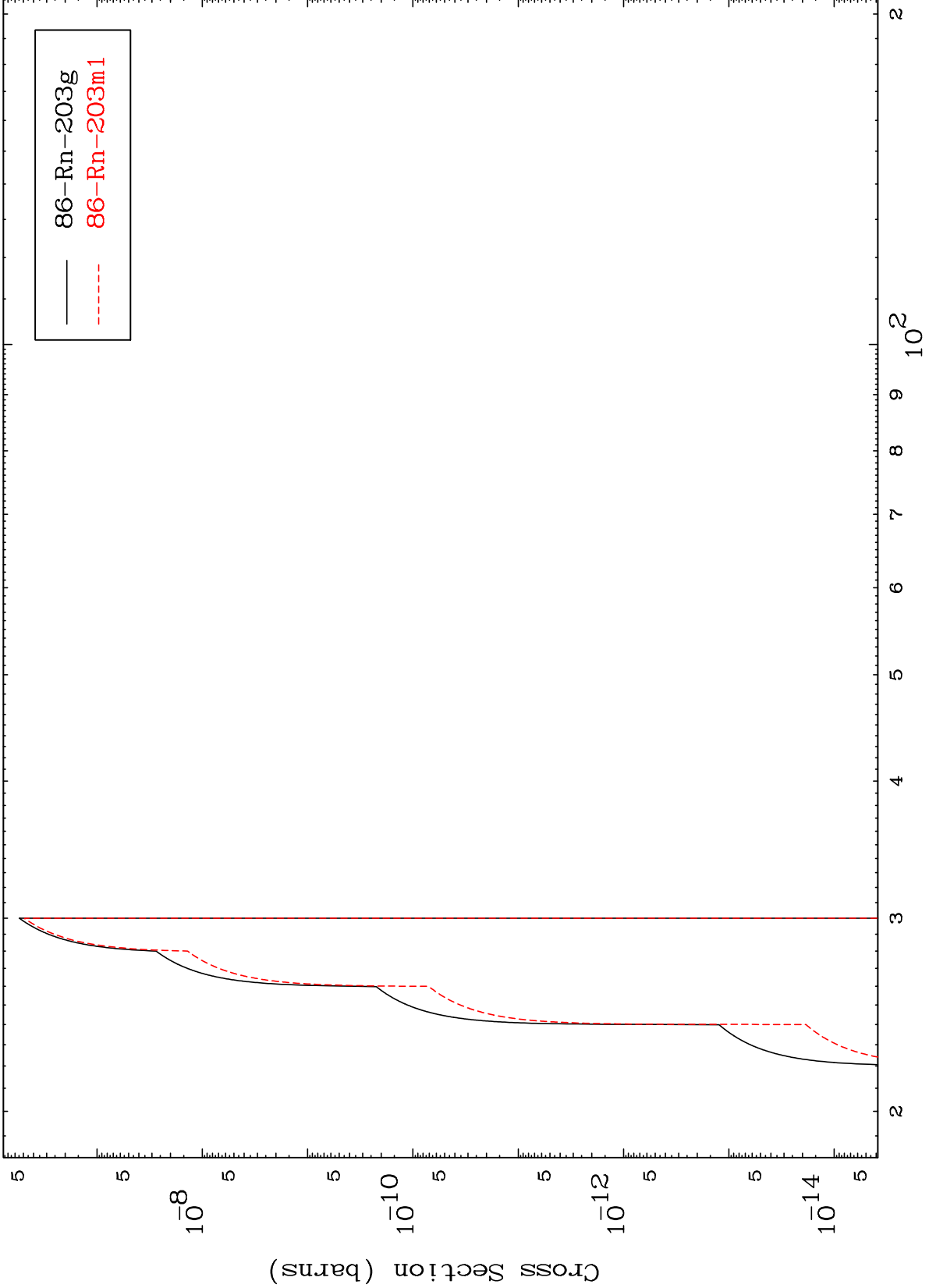
$^{86}\text{Rn-206}$

MAT 8610

(n,n') t

86-Rn-206

Radionuclide Production Cross Section

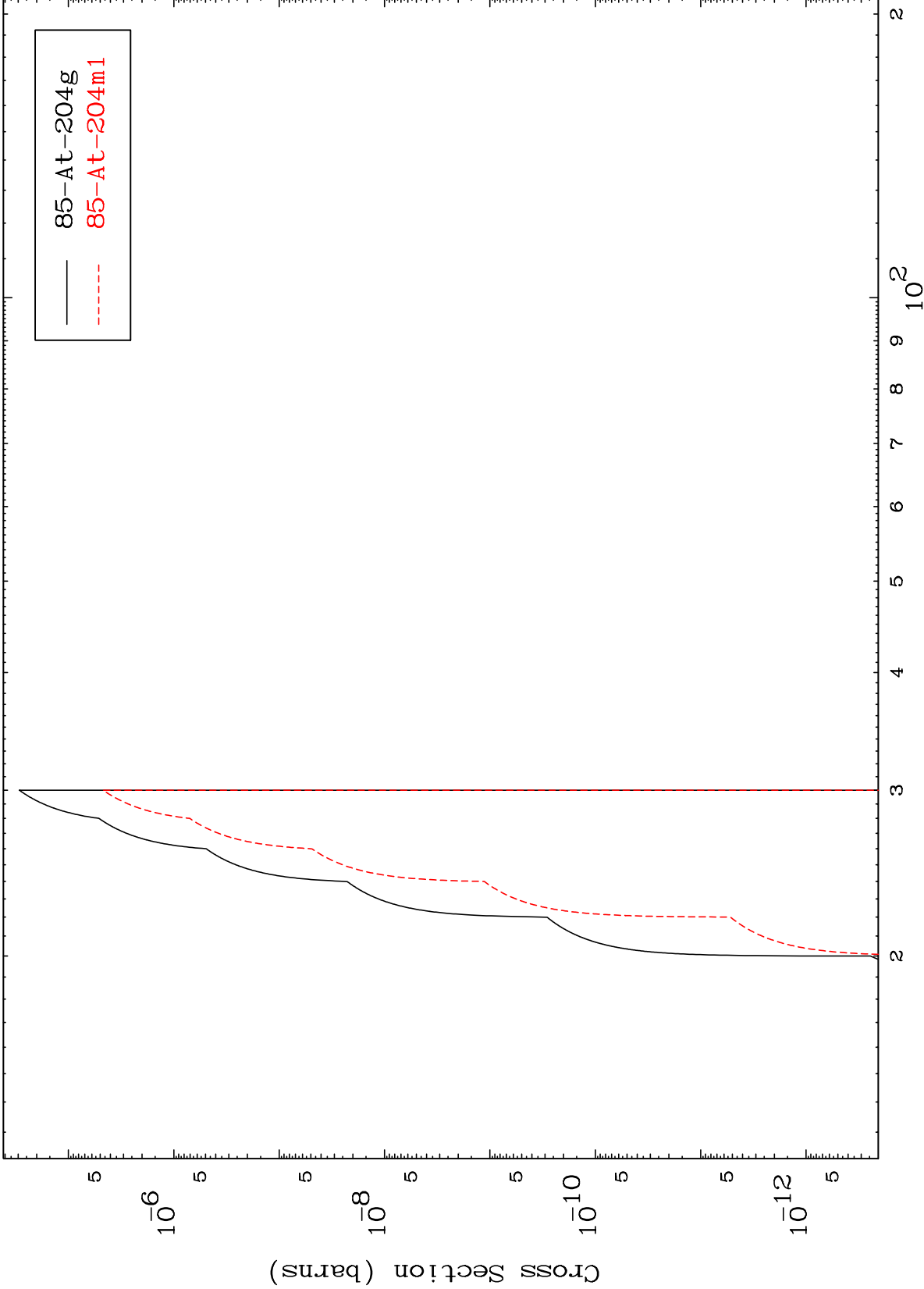


20

Incident Energy (MeV)

86-Rn-206

Radionuclide Production Cross Section



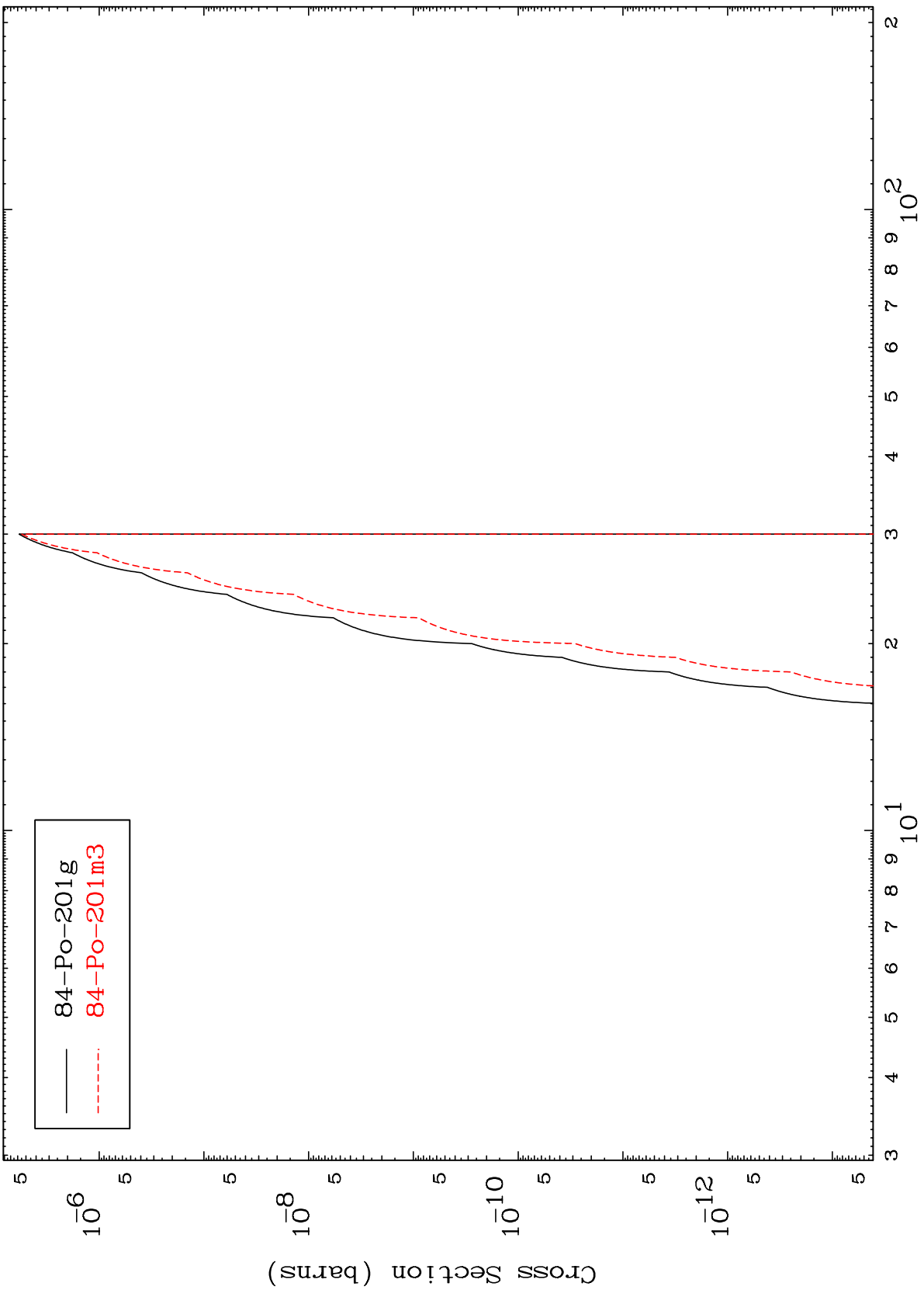
85-At-204g
85-At-204m1

MAT 8610

(n,n') p α

86-Rn-206

Radionuclide Production Cross Section



22

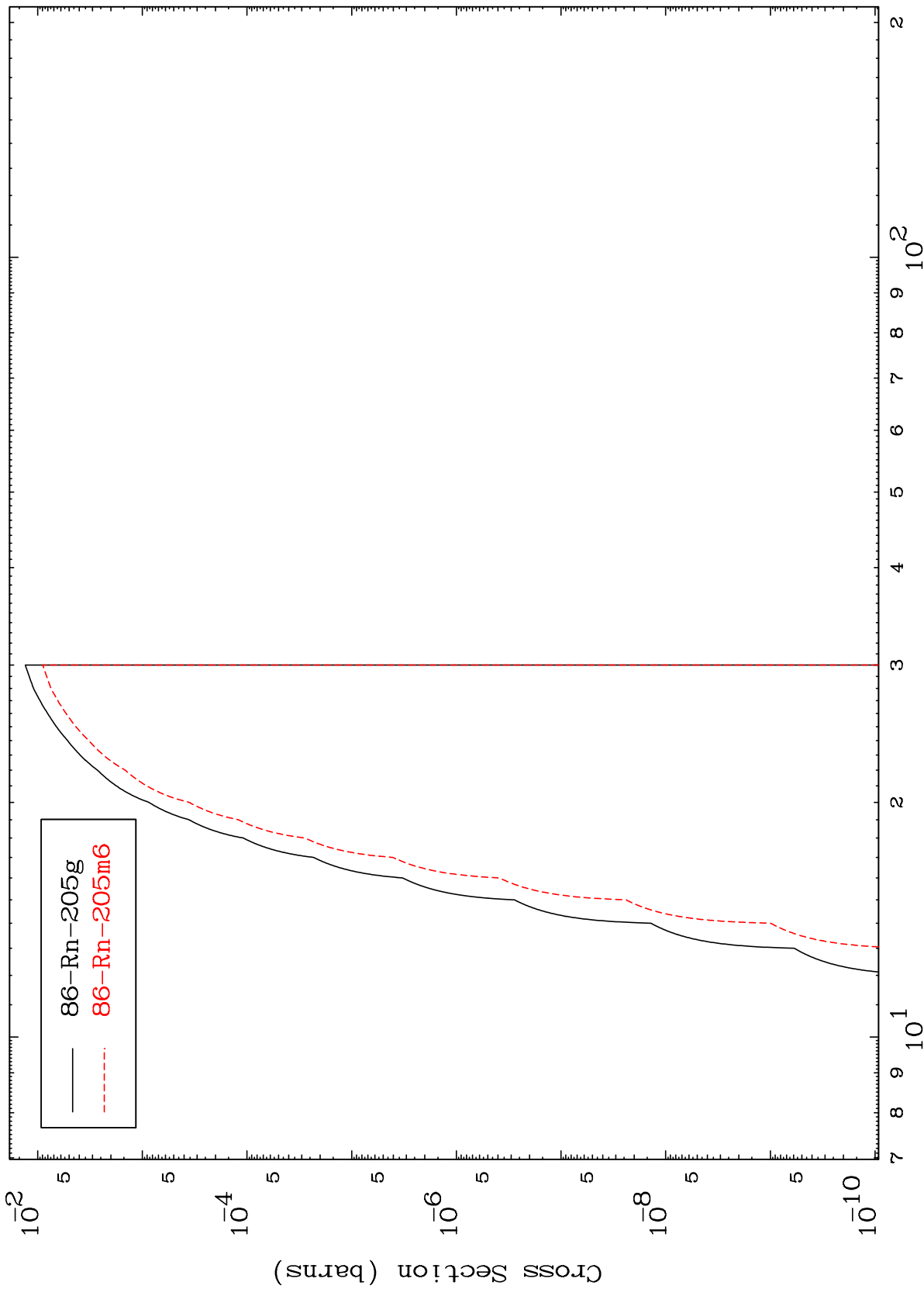
Incident Energy (MeV)

86-Rn-206

MAT 8610

86-Rn-206

(n,d)
Radionuclide Production Cross Section



Incident Energy (MeV)

86-Rn-206

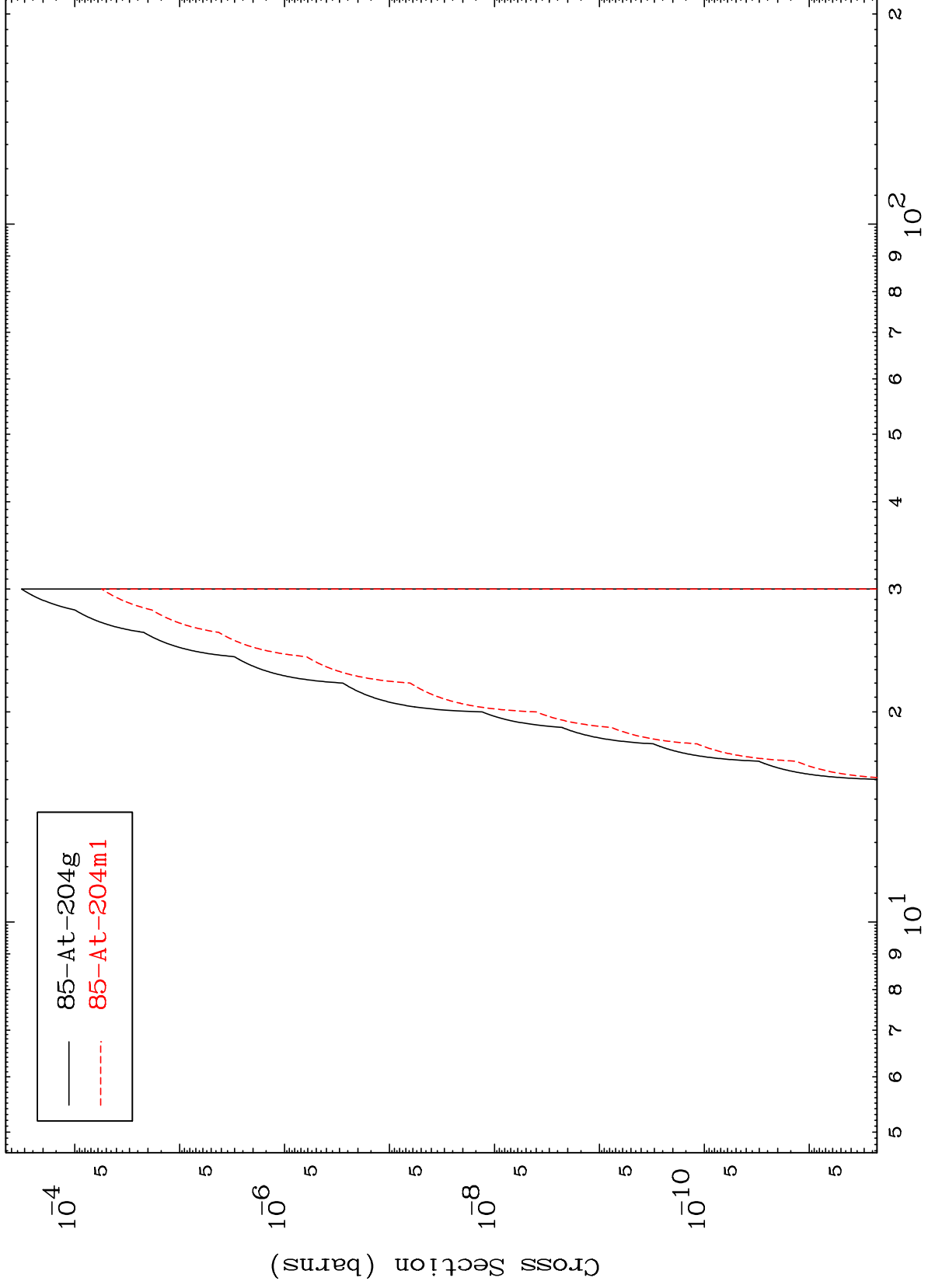
23

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(n,He-3)

86-Rn-206

Radionuclide Production Cross Section



85-At-204g
85-At-204m1

24

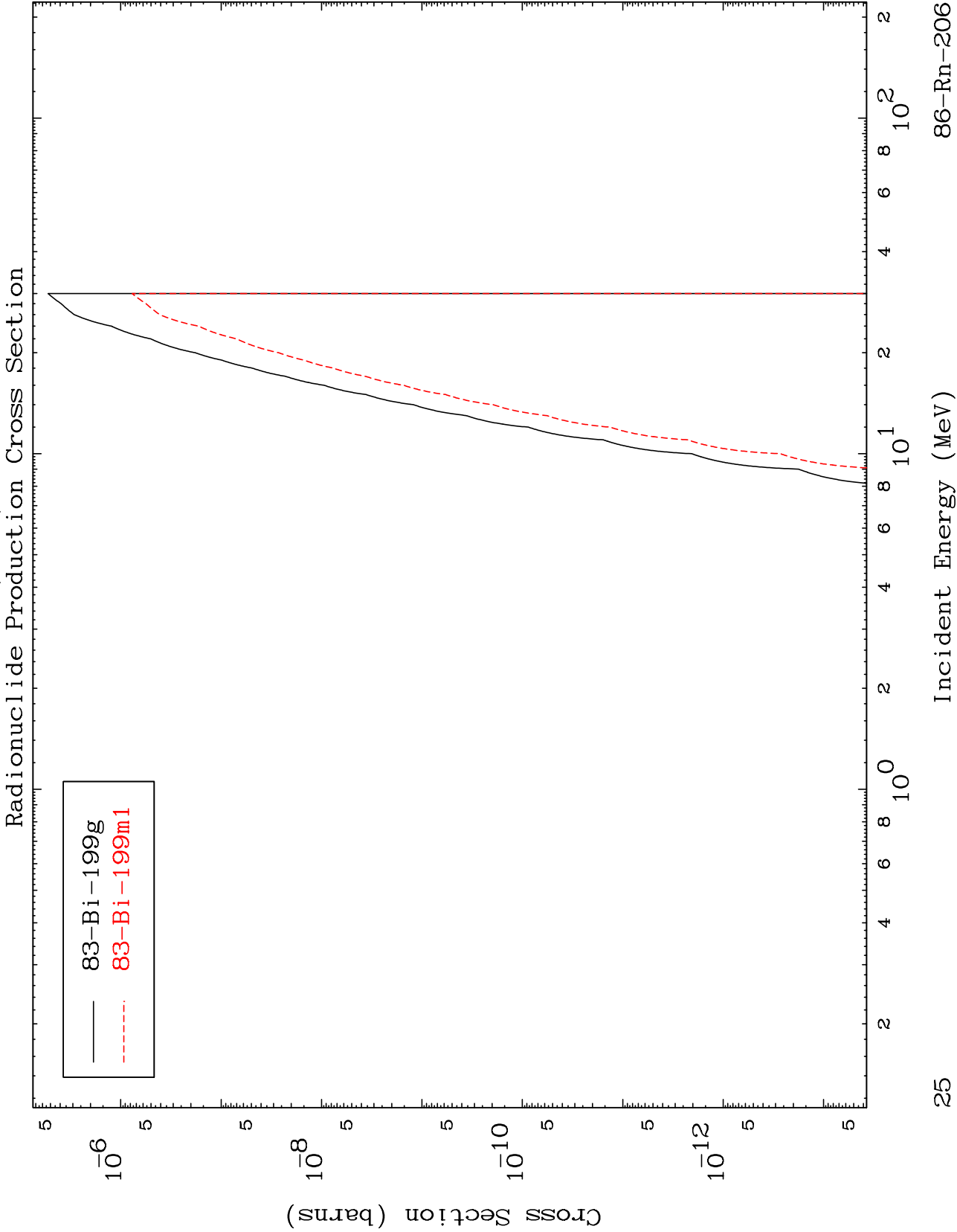
Incident Energy (MeV)

86-Rn-206

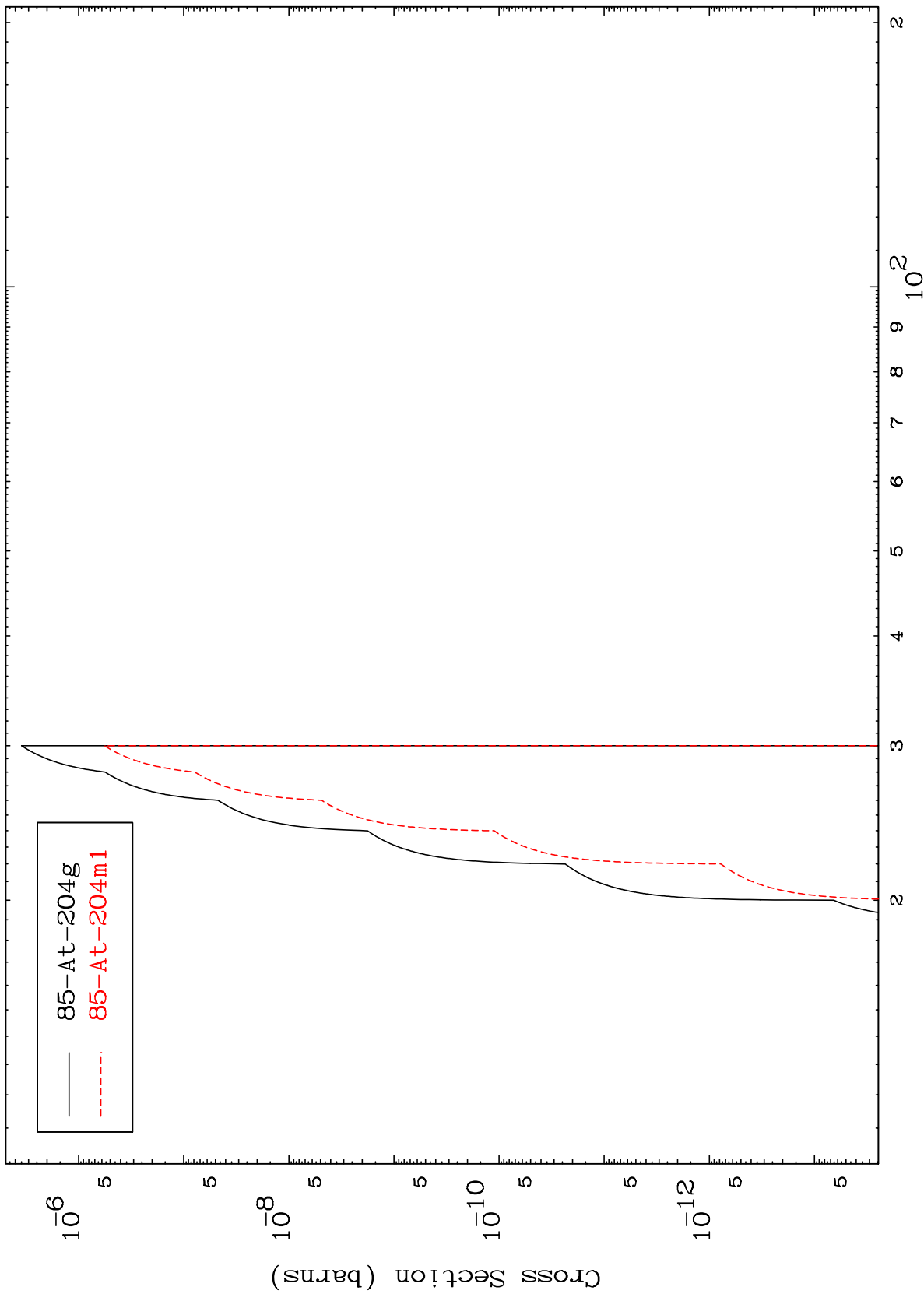
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(n,2α)

86-Rn-206



Radionuclide Production Cross Section

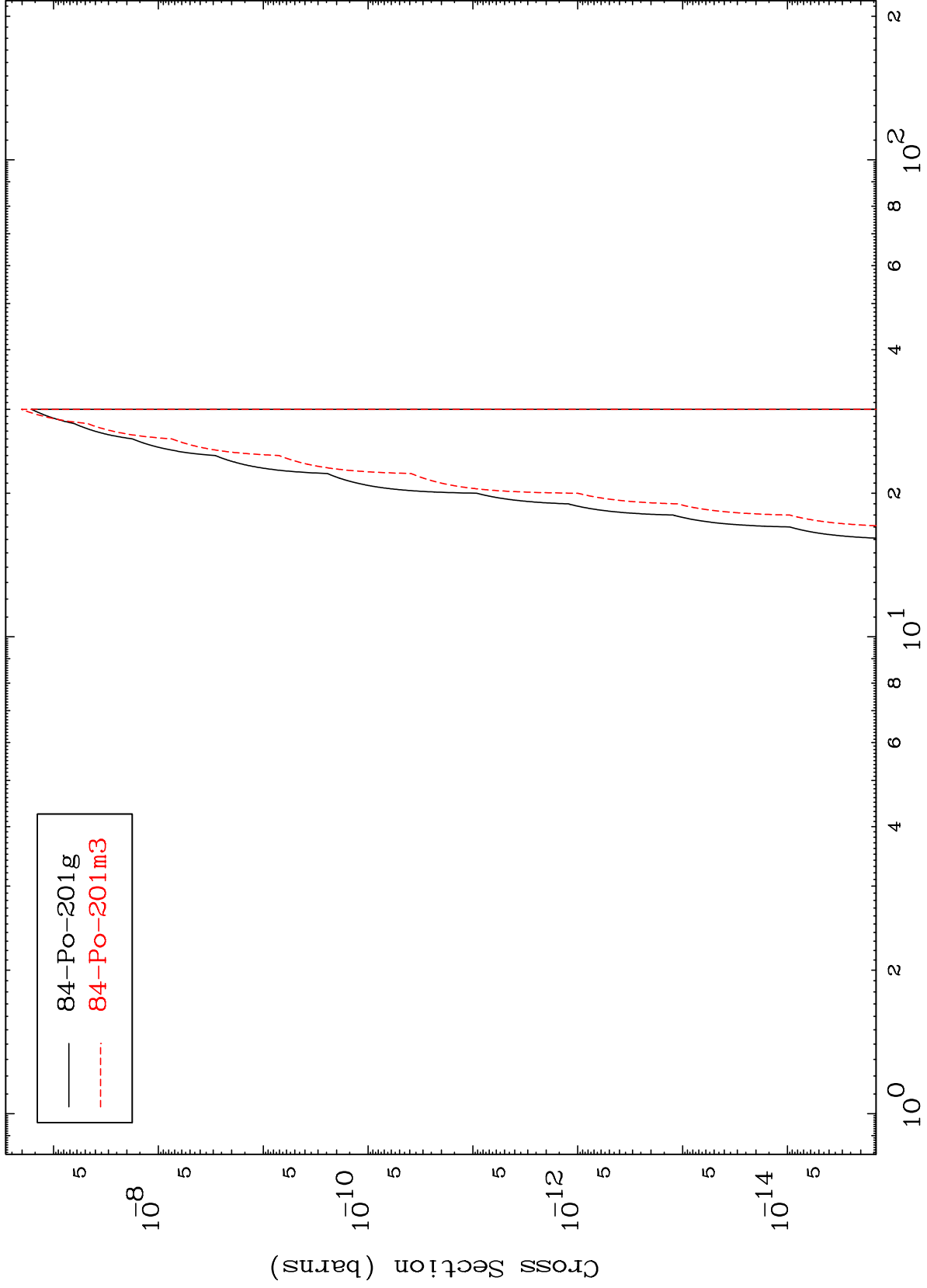


MAT 8610

(n,d) α

86-Rn-206

Radionuclide Production Cross Section



27

Incident Energy (MeV)

86-Rn-206