

Program EVALPLOT
(Version 2021-1)

by

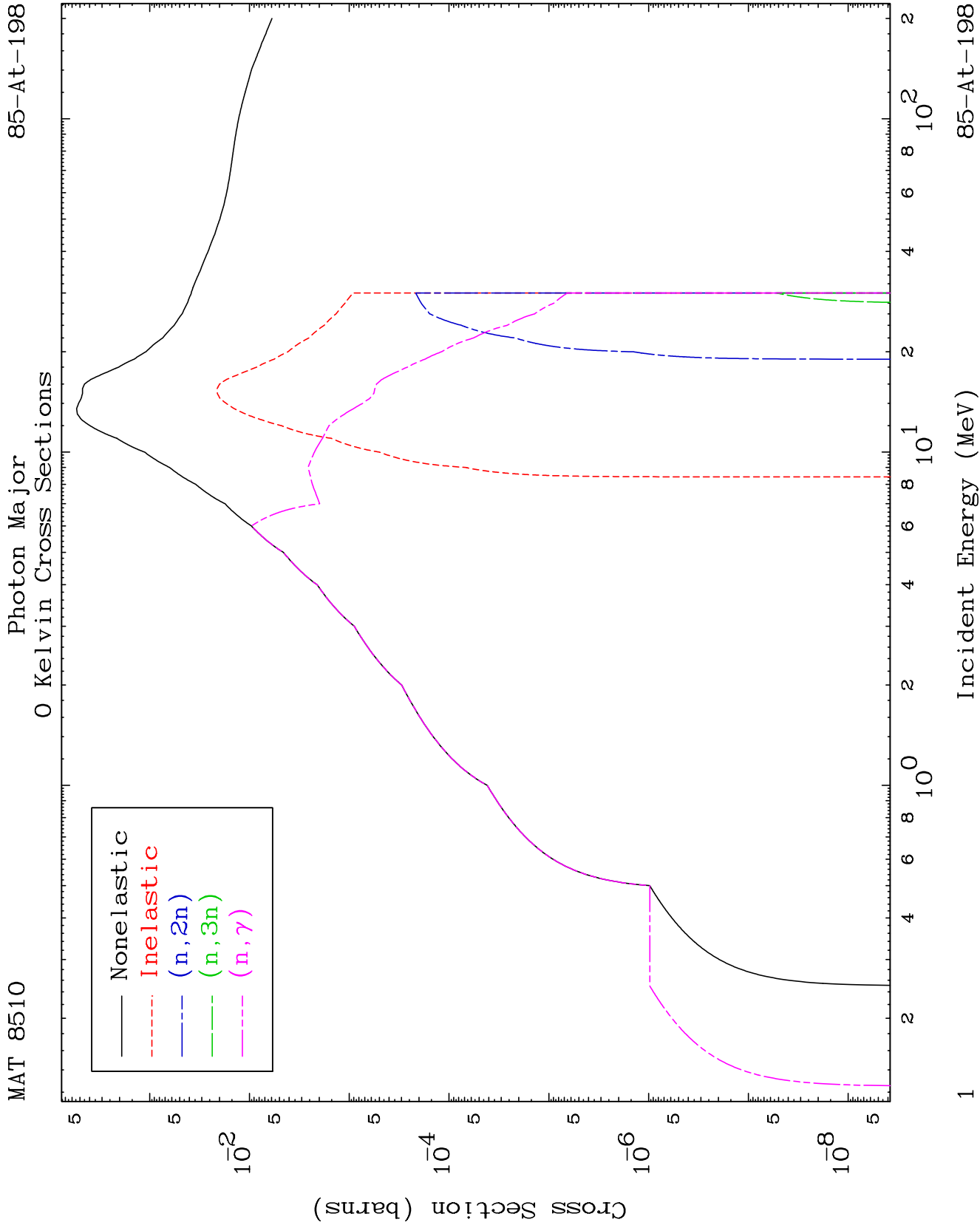
Dermott E. Cullen
(Present Contact Information)

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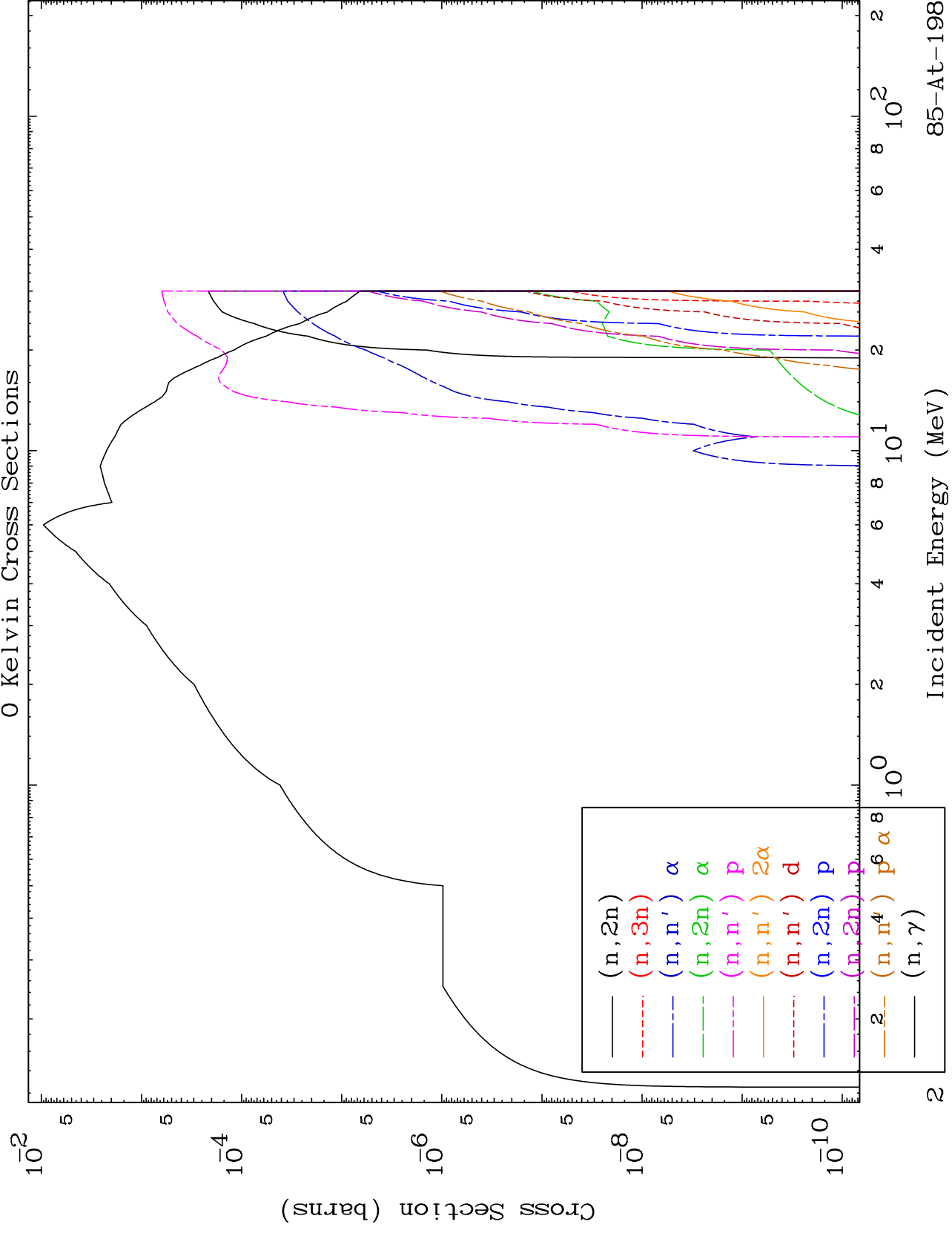
Press Mouse Button to Start

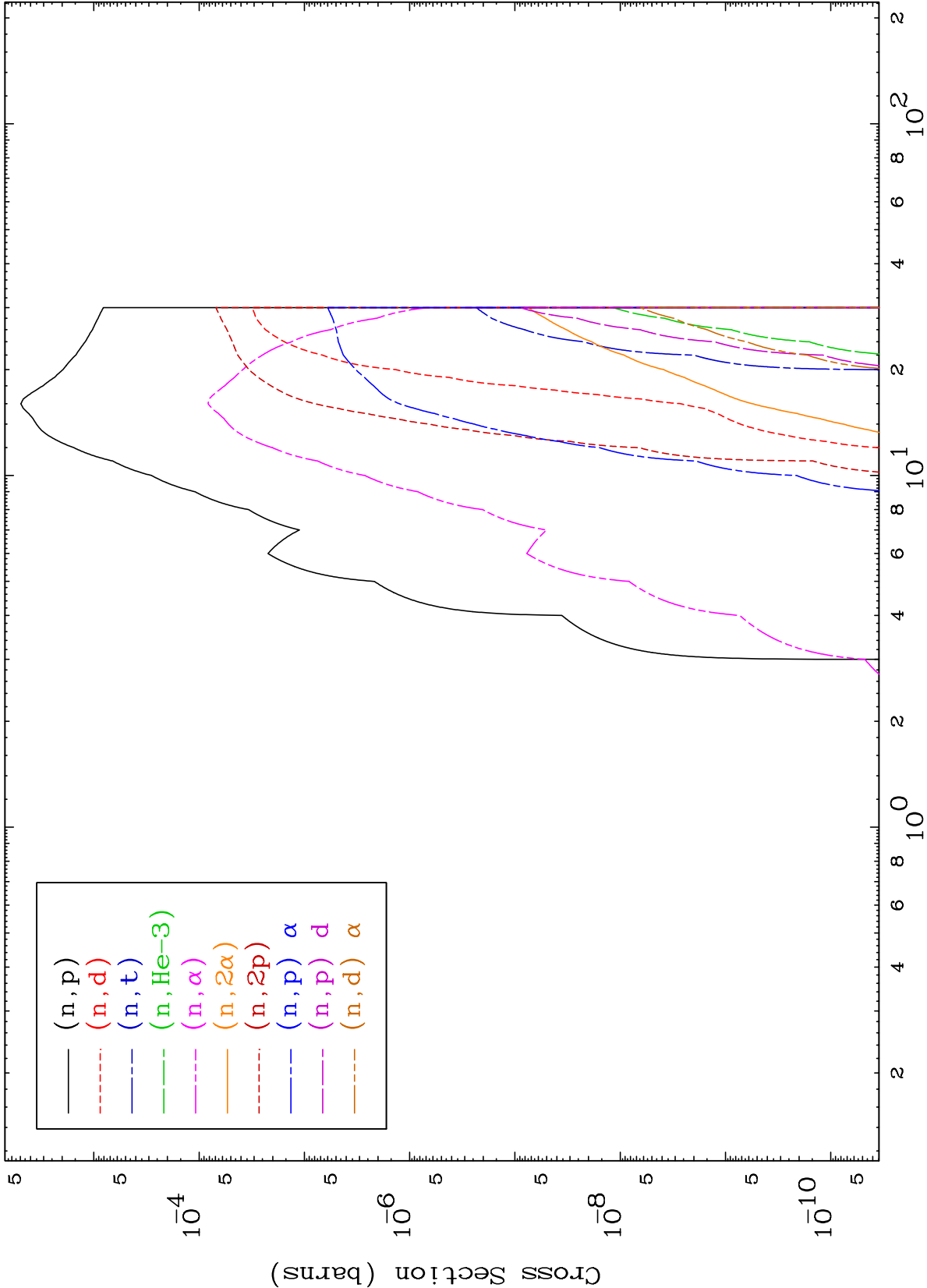


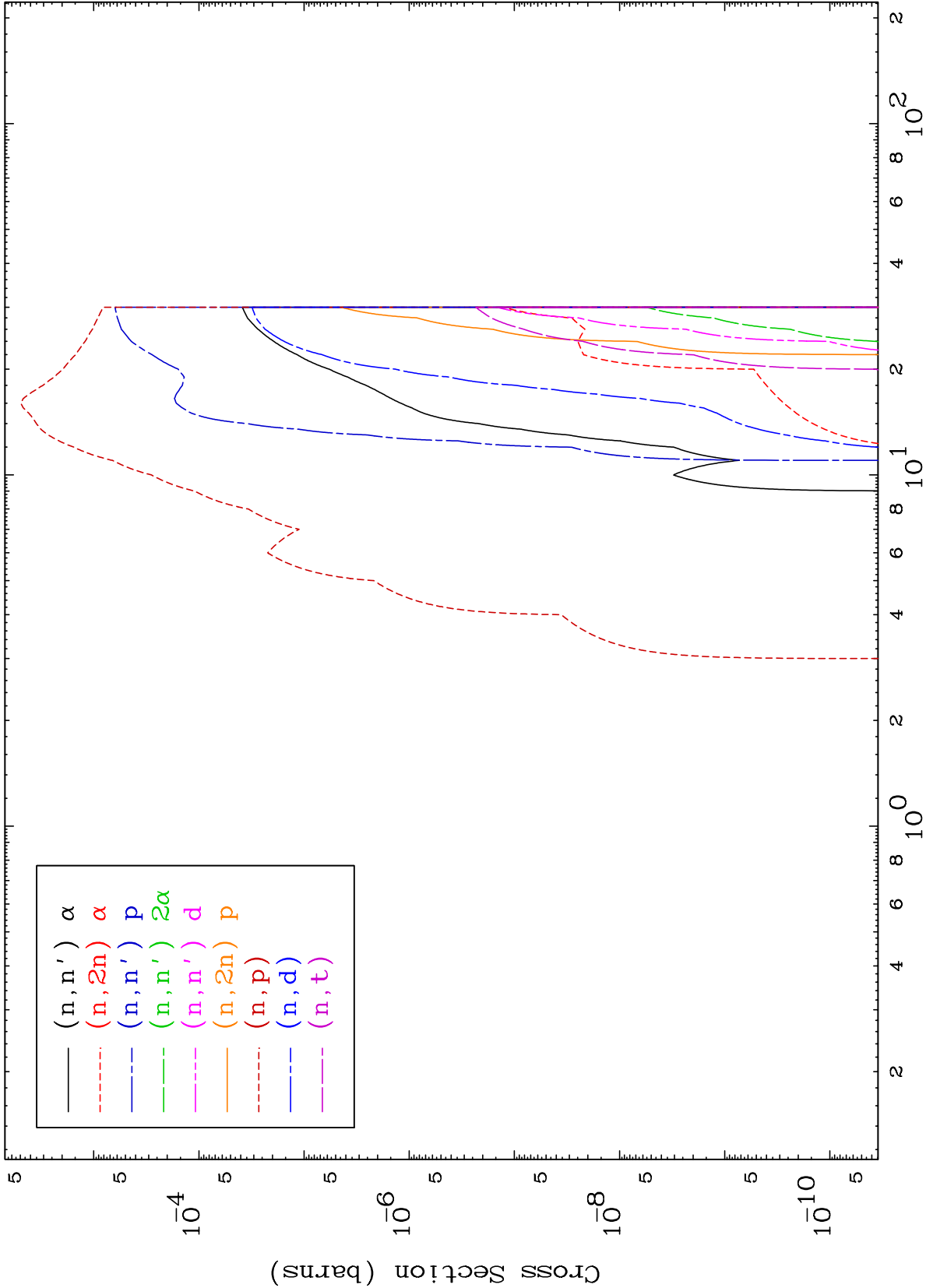
MAT 8510

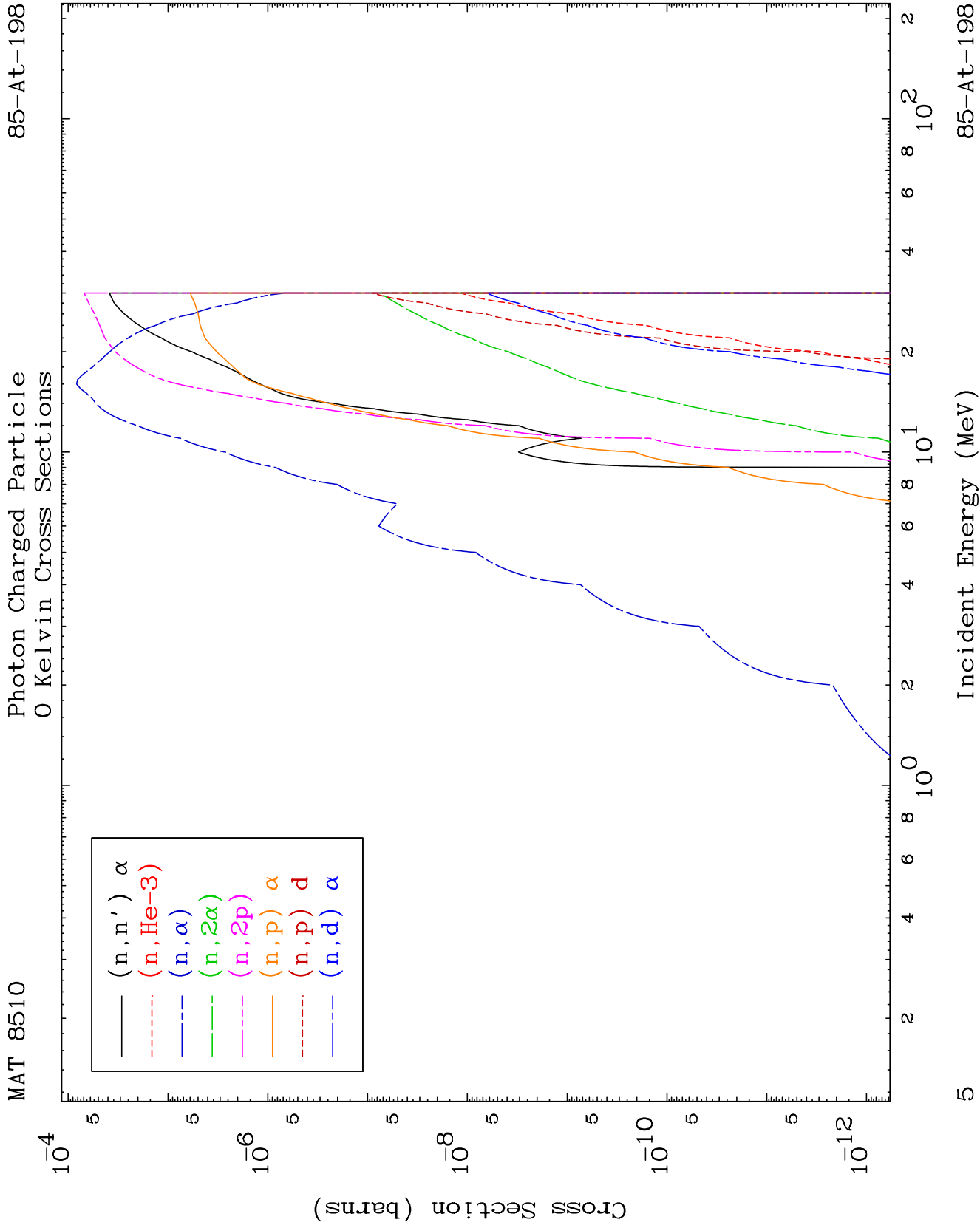
Photon Neutron Absorption 0 Kelvin Cross Sections

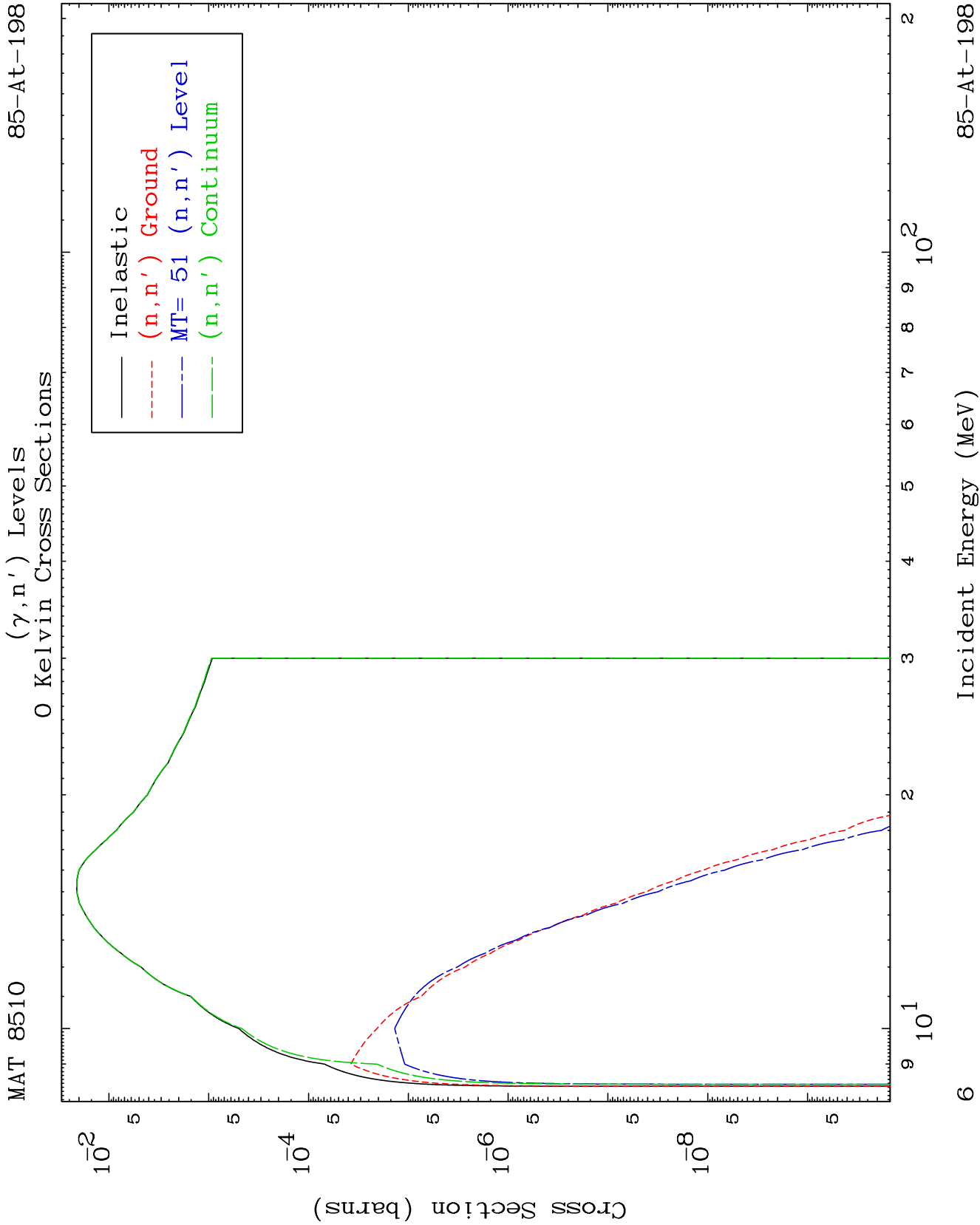
85-At-198







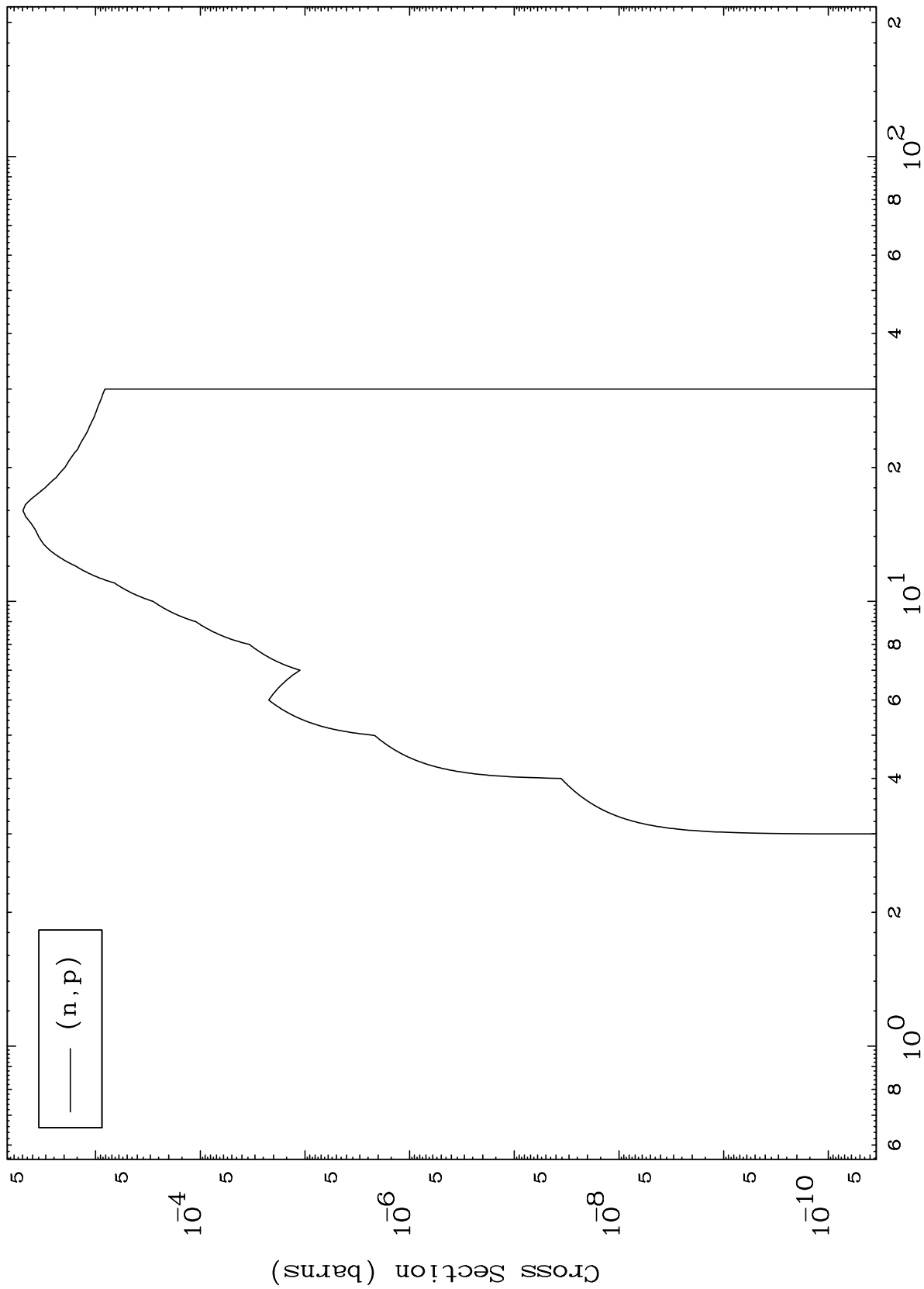




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(γ ,p) Levels
0 Kelvin Cross Sections



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Incident Energy (MeV)

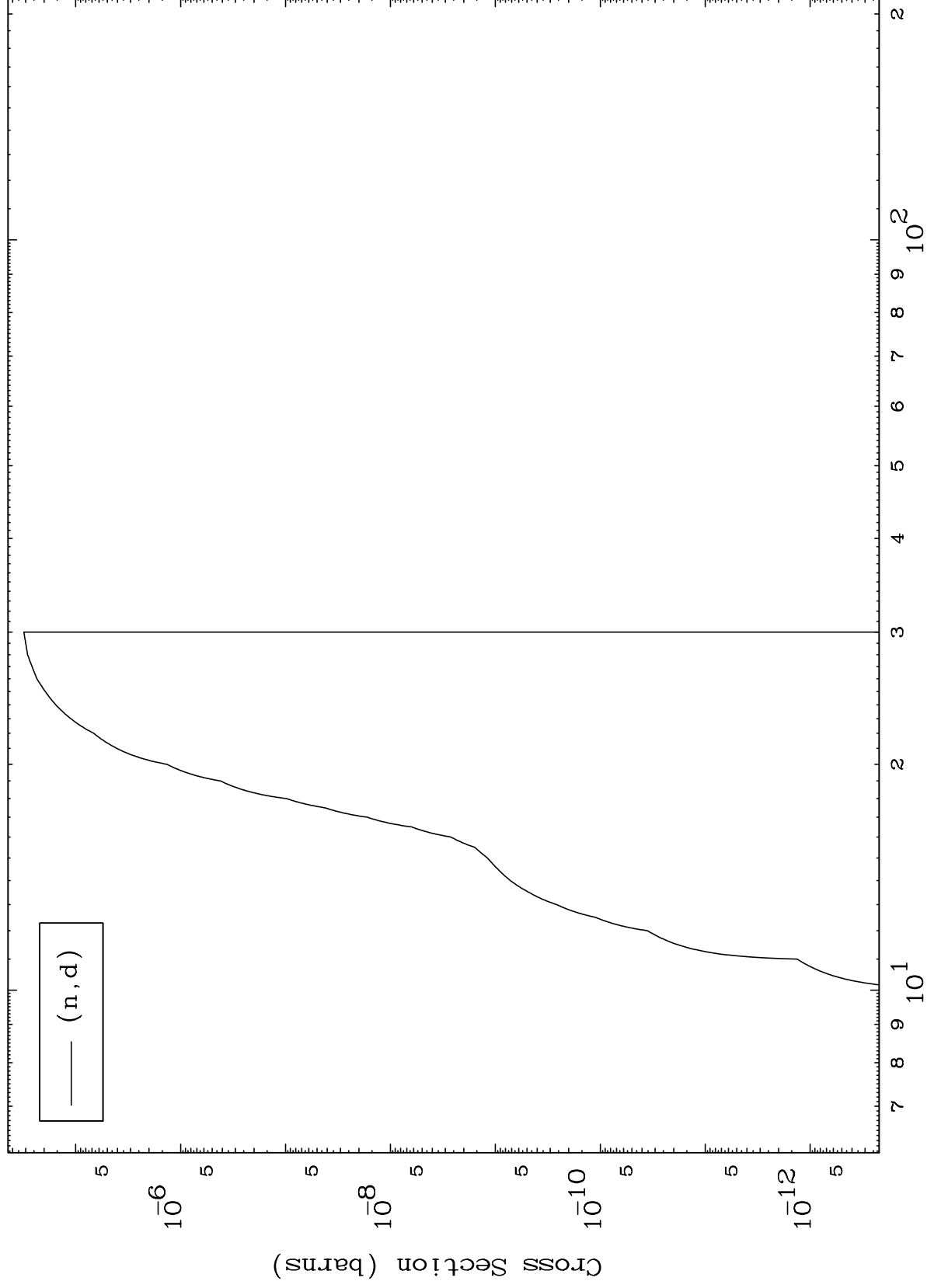
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(γ, d) Levels

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0 Kelvin Cross Sections

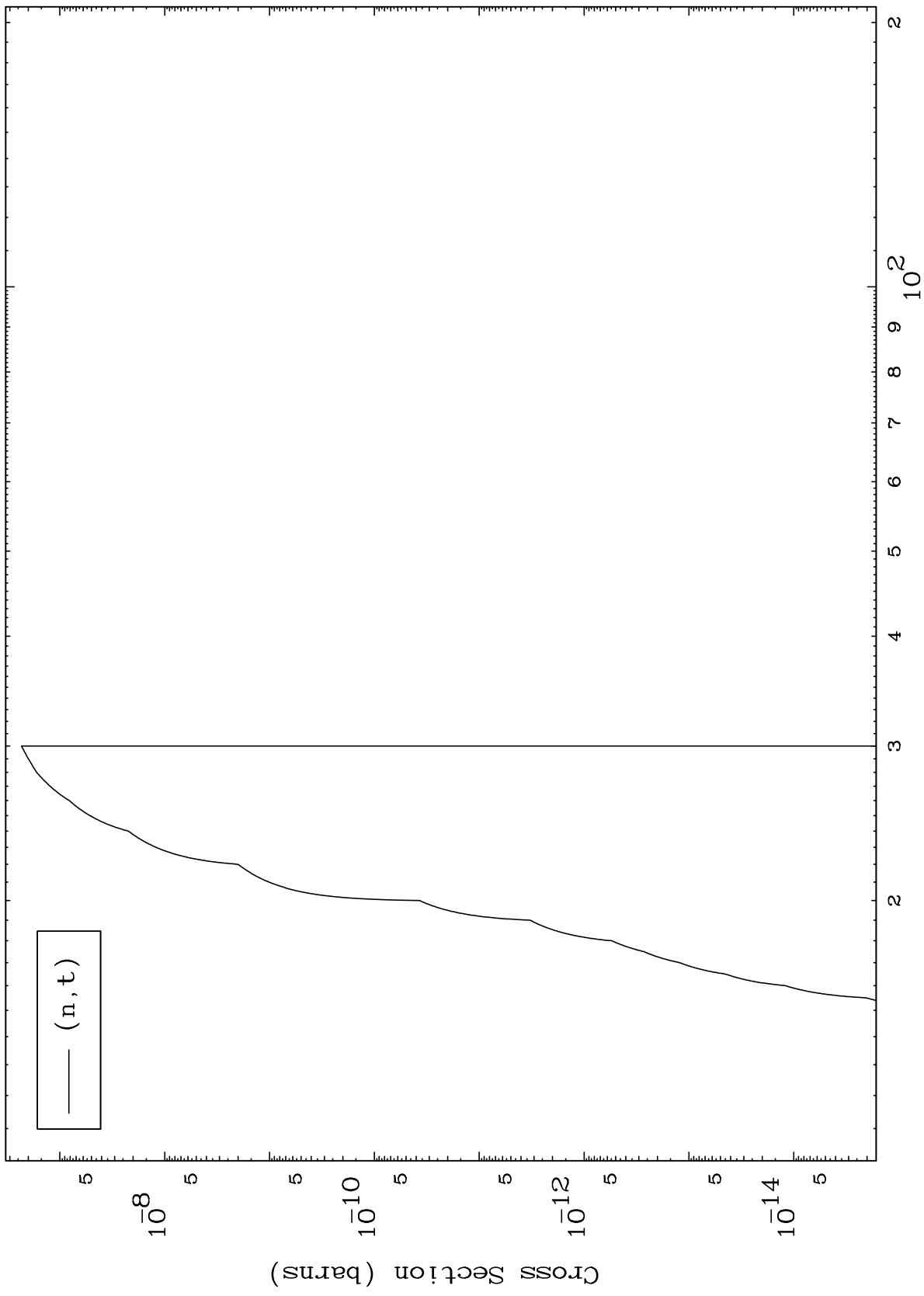


8

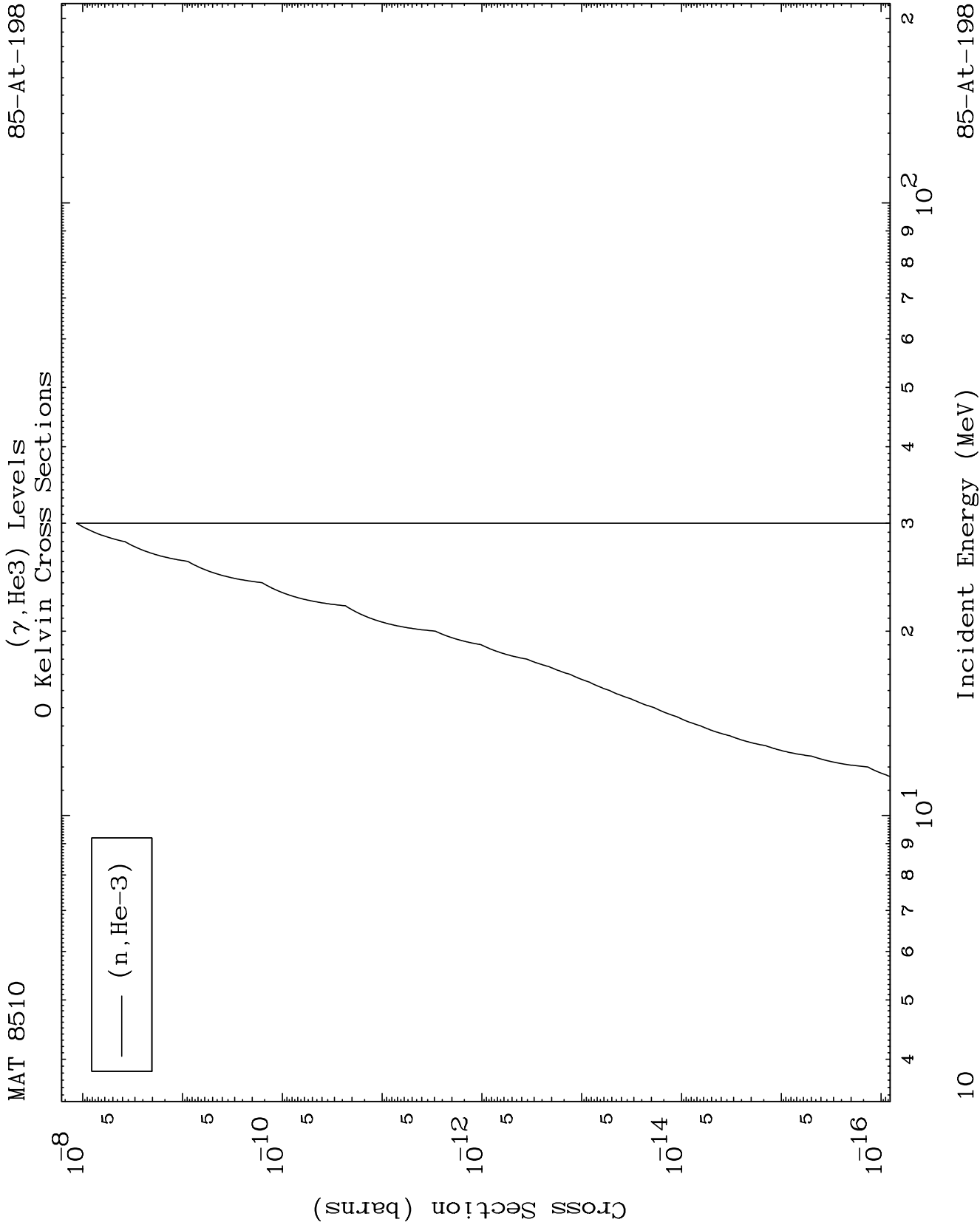
Incident Energy (MeV)

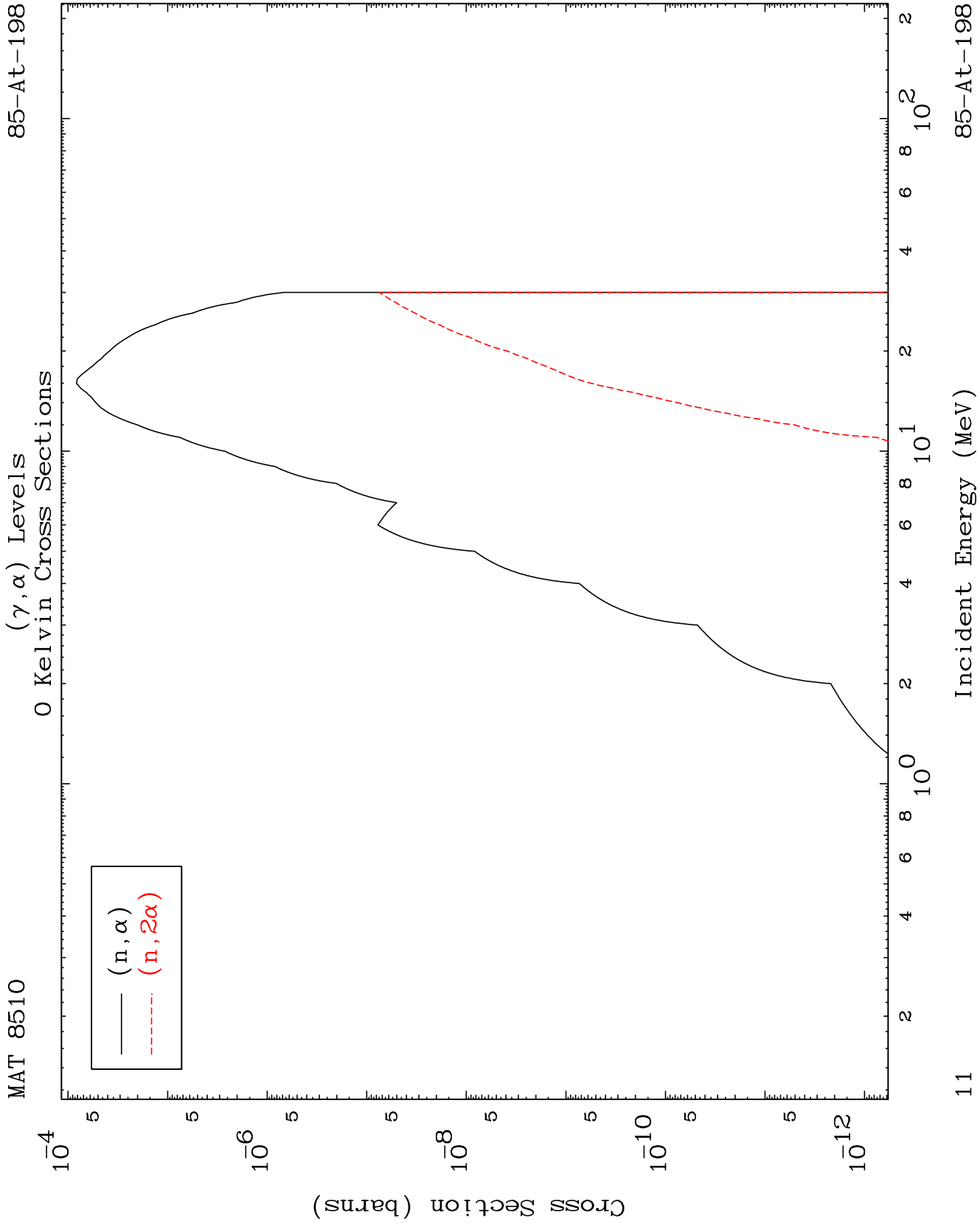
85-At-198

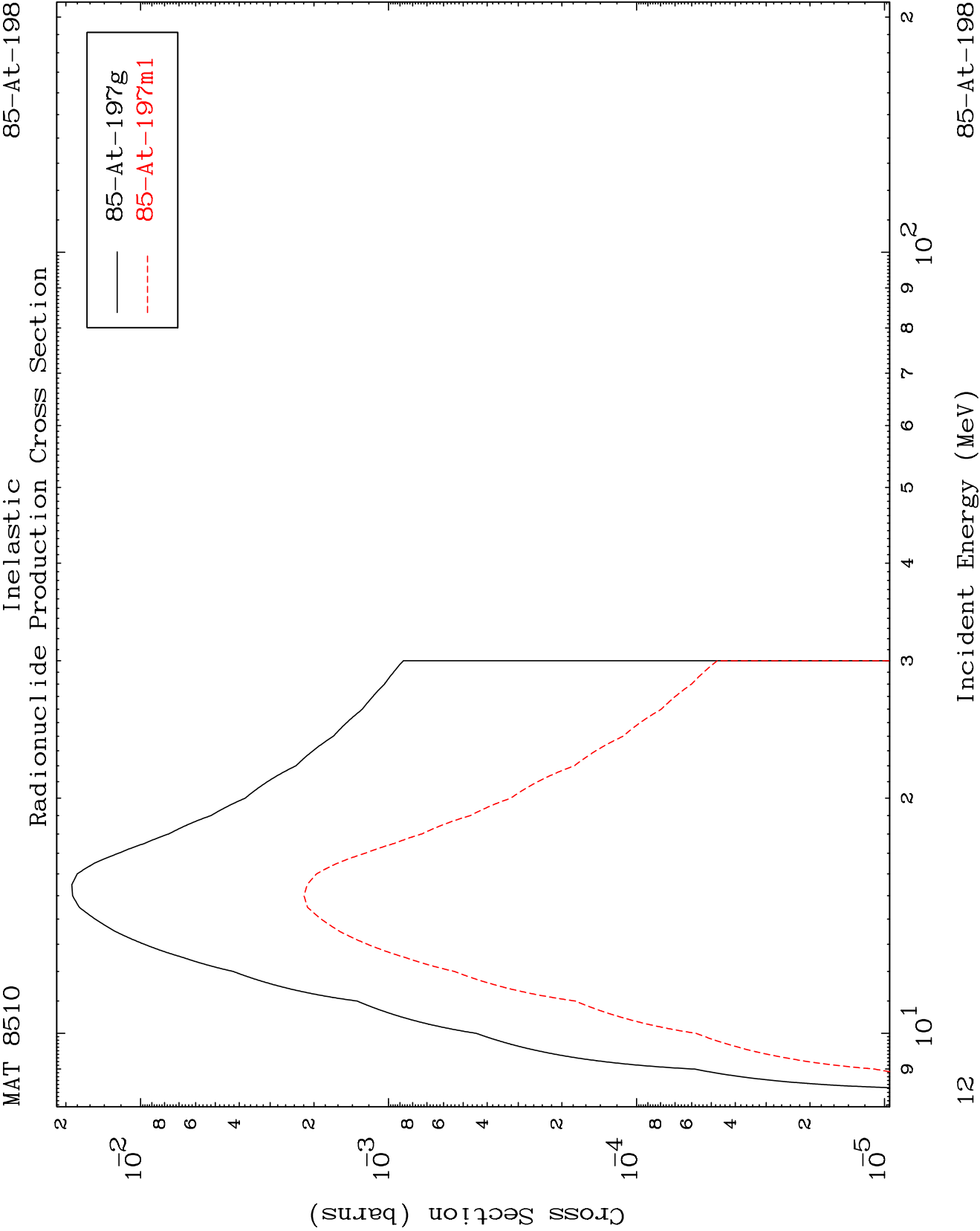
(γ, t) Levels
0 Kelvin Cross Sections

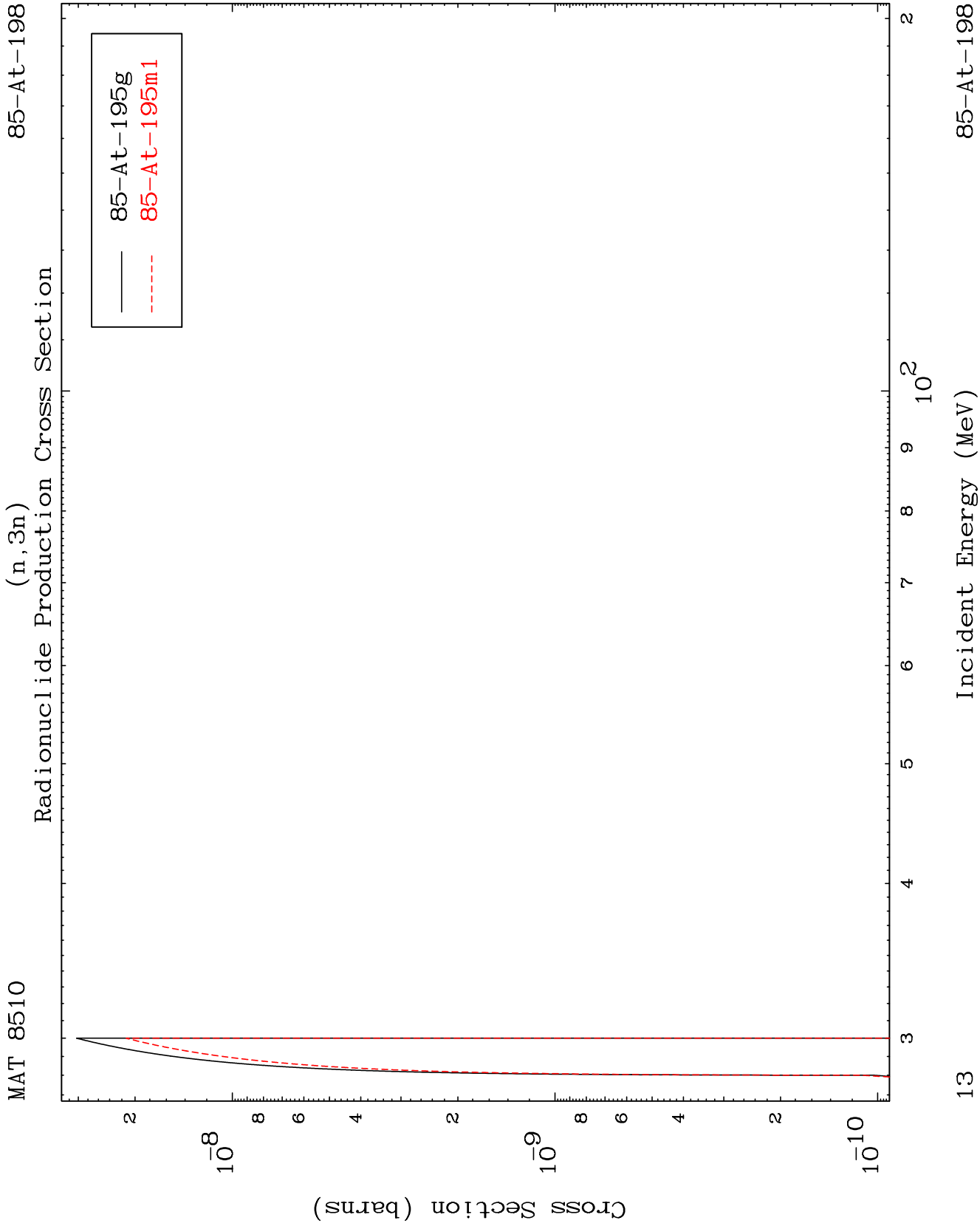


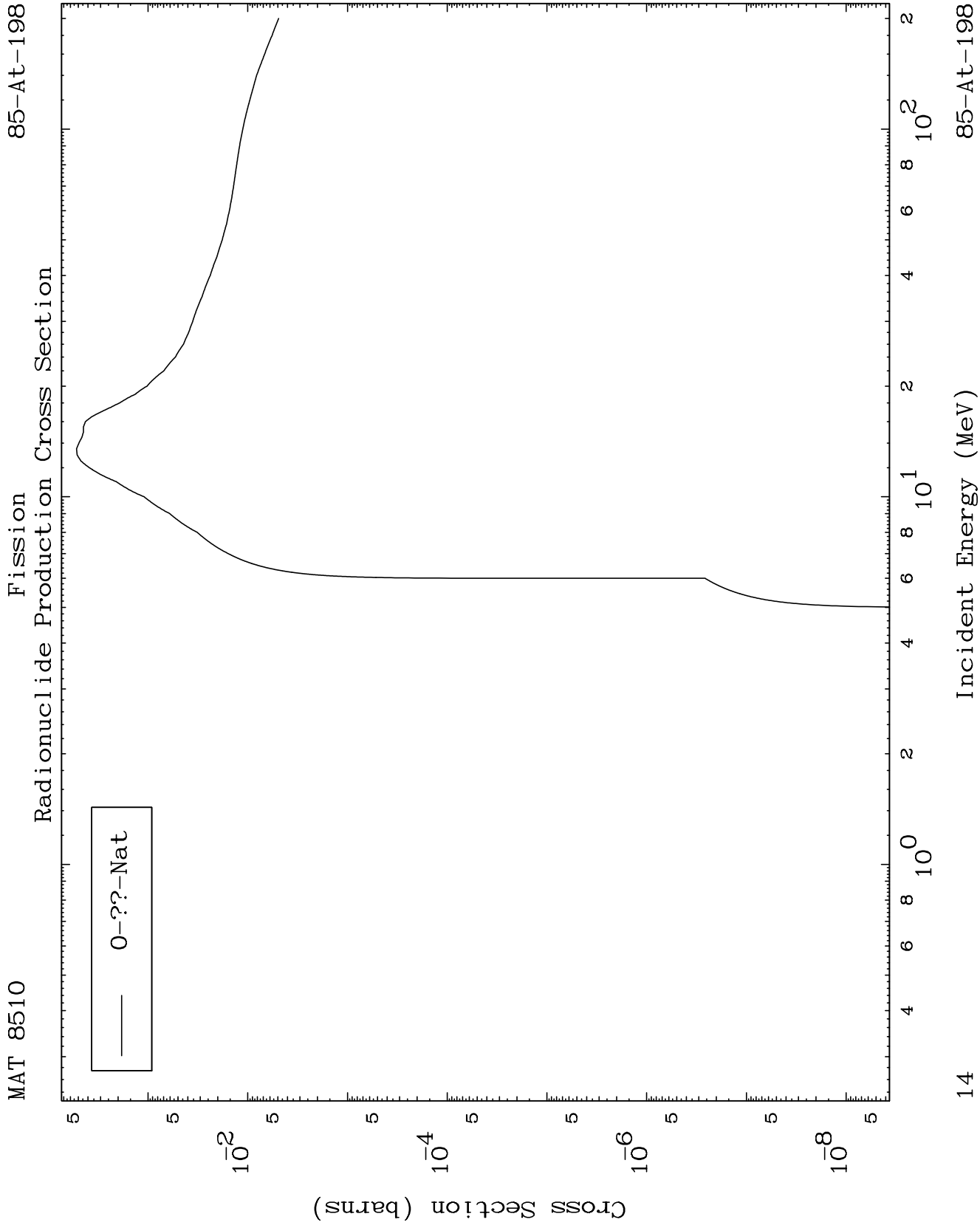
Incident Energy (MeV)







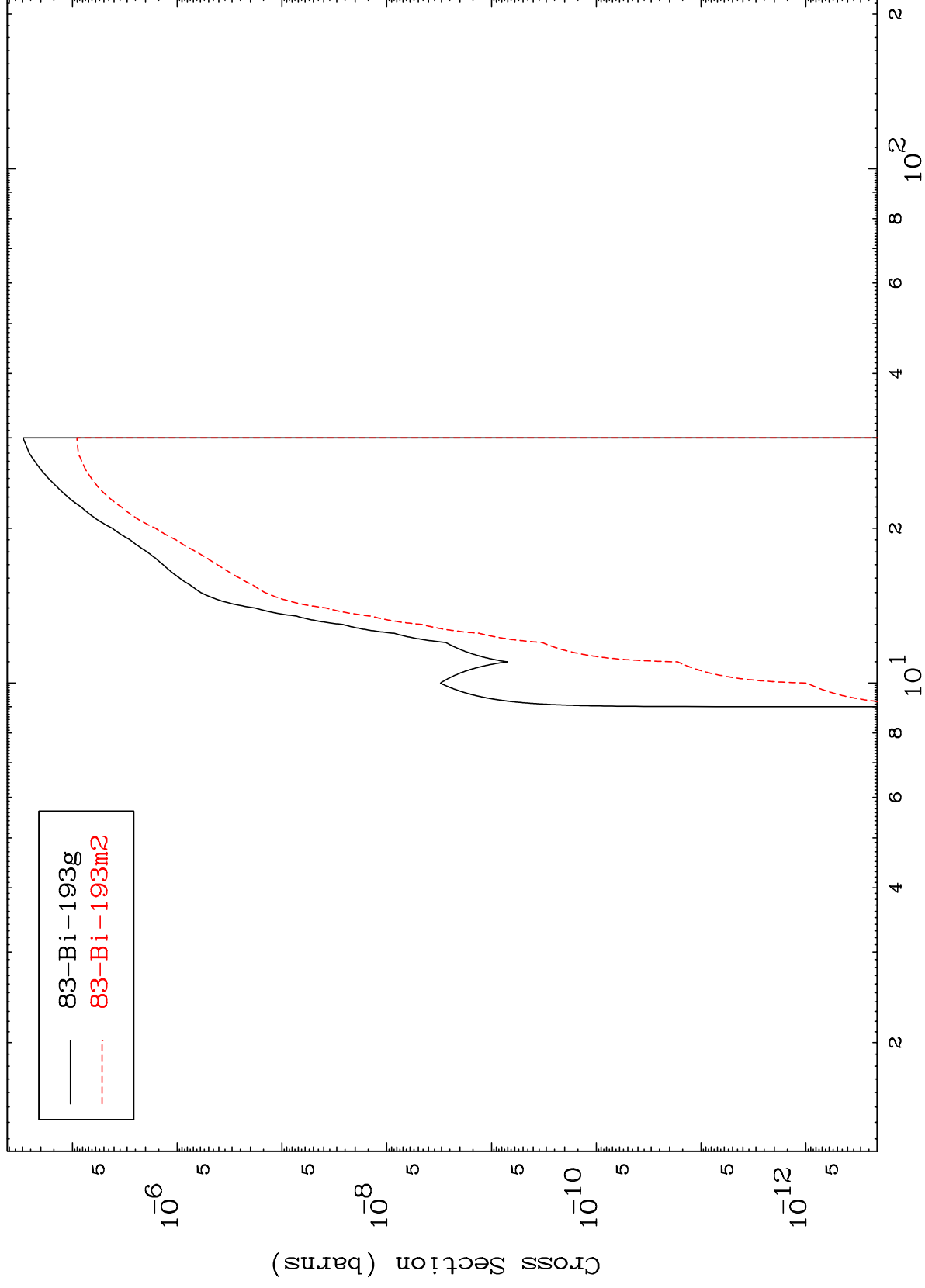




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Radionuclide Production Cross Section
(n,n') α



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Incident Energy (MeV)

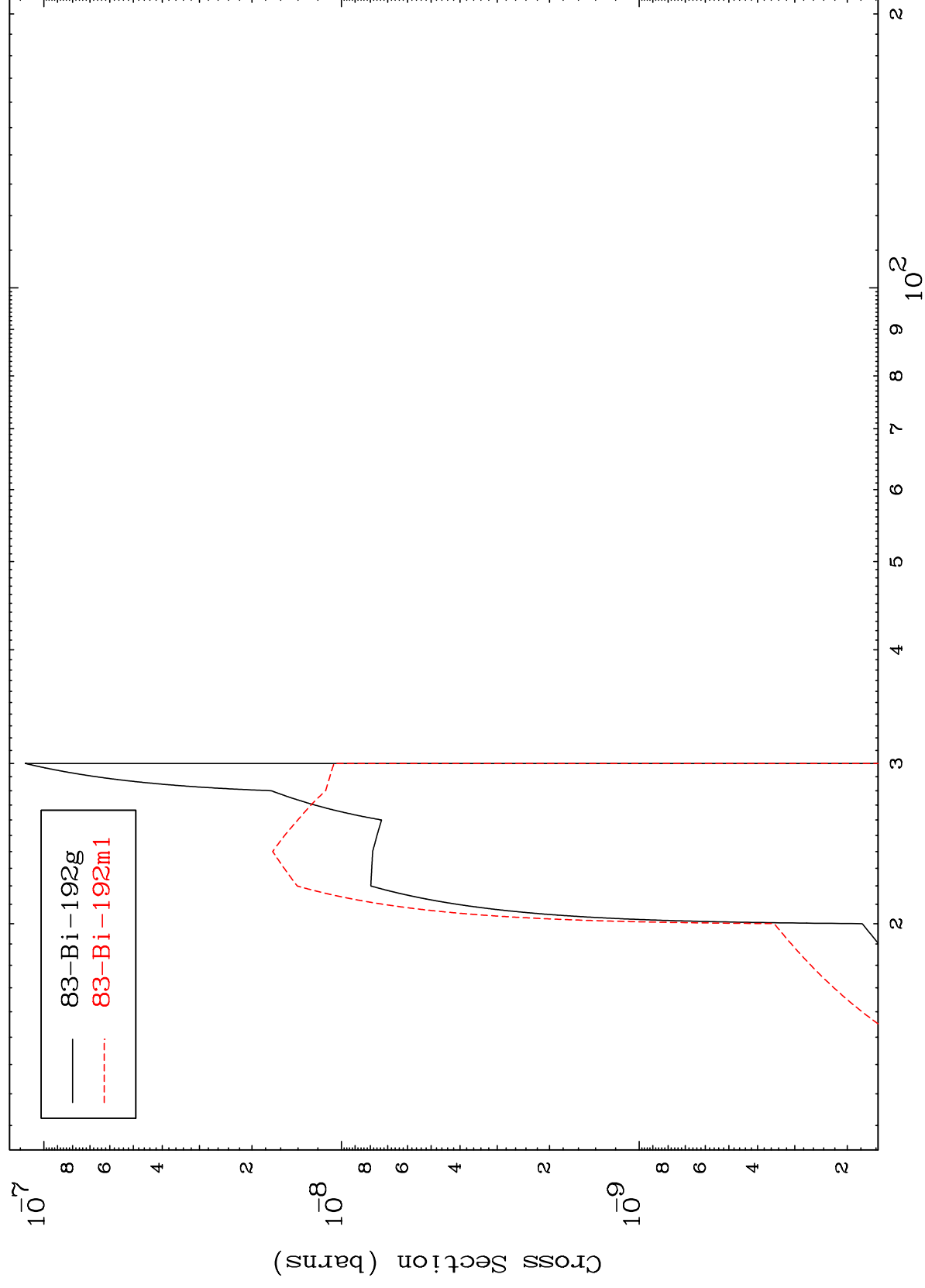
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$$(n, 2n) \propto$$

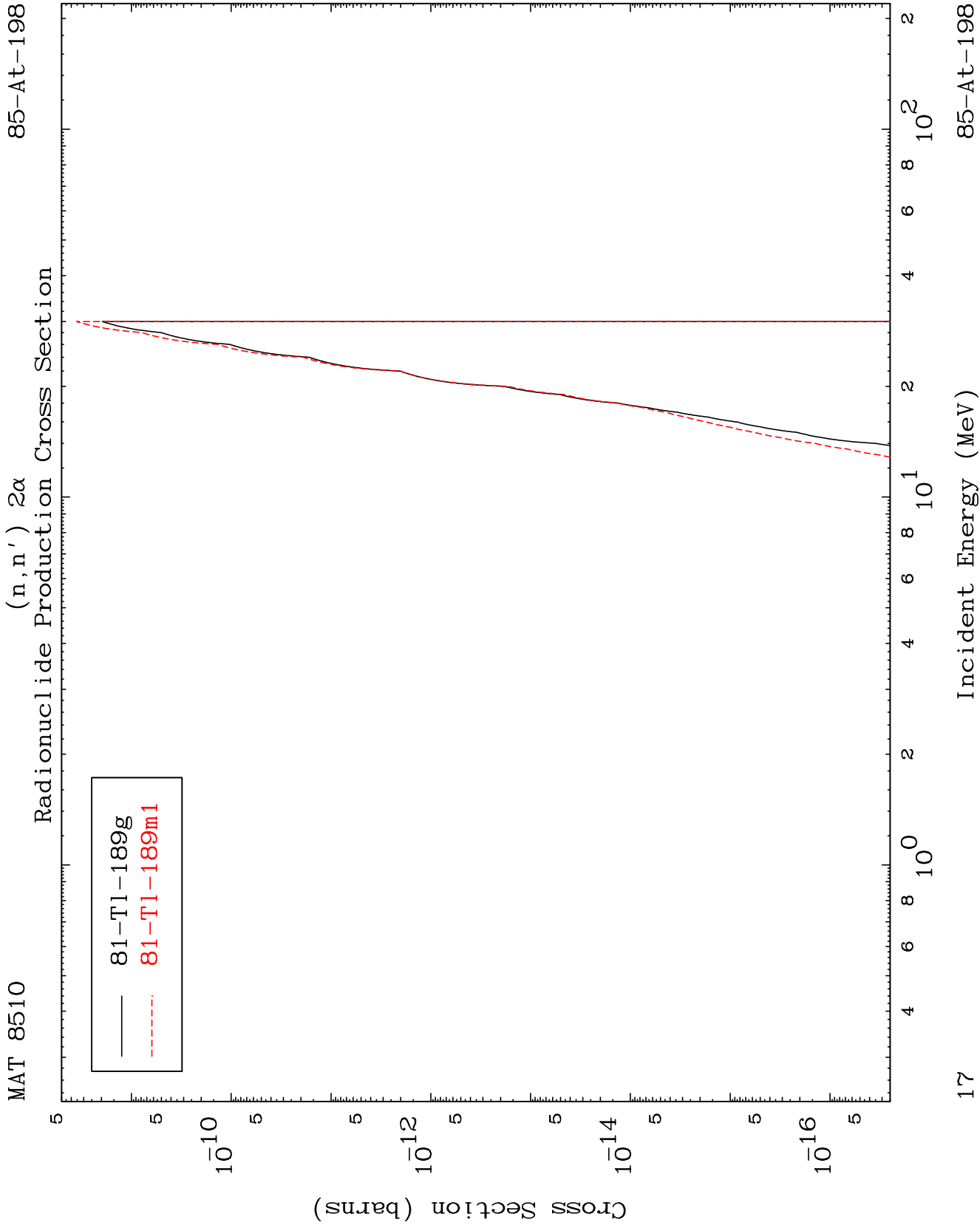
Radionuclide Production Cross Section



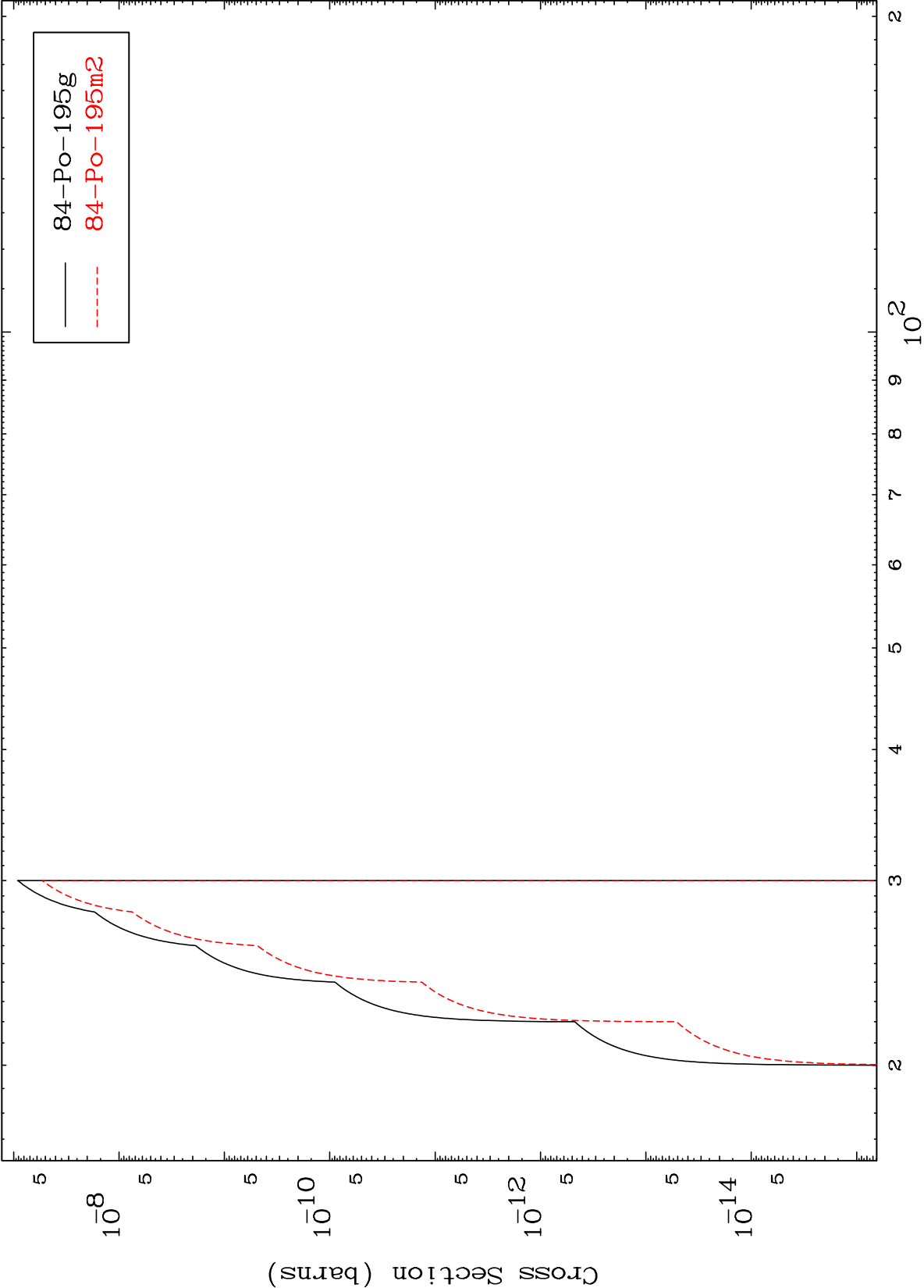
16

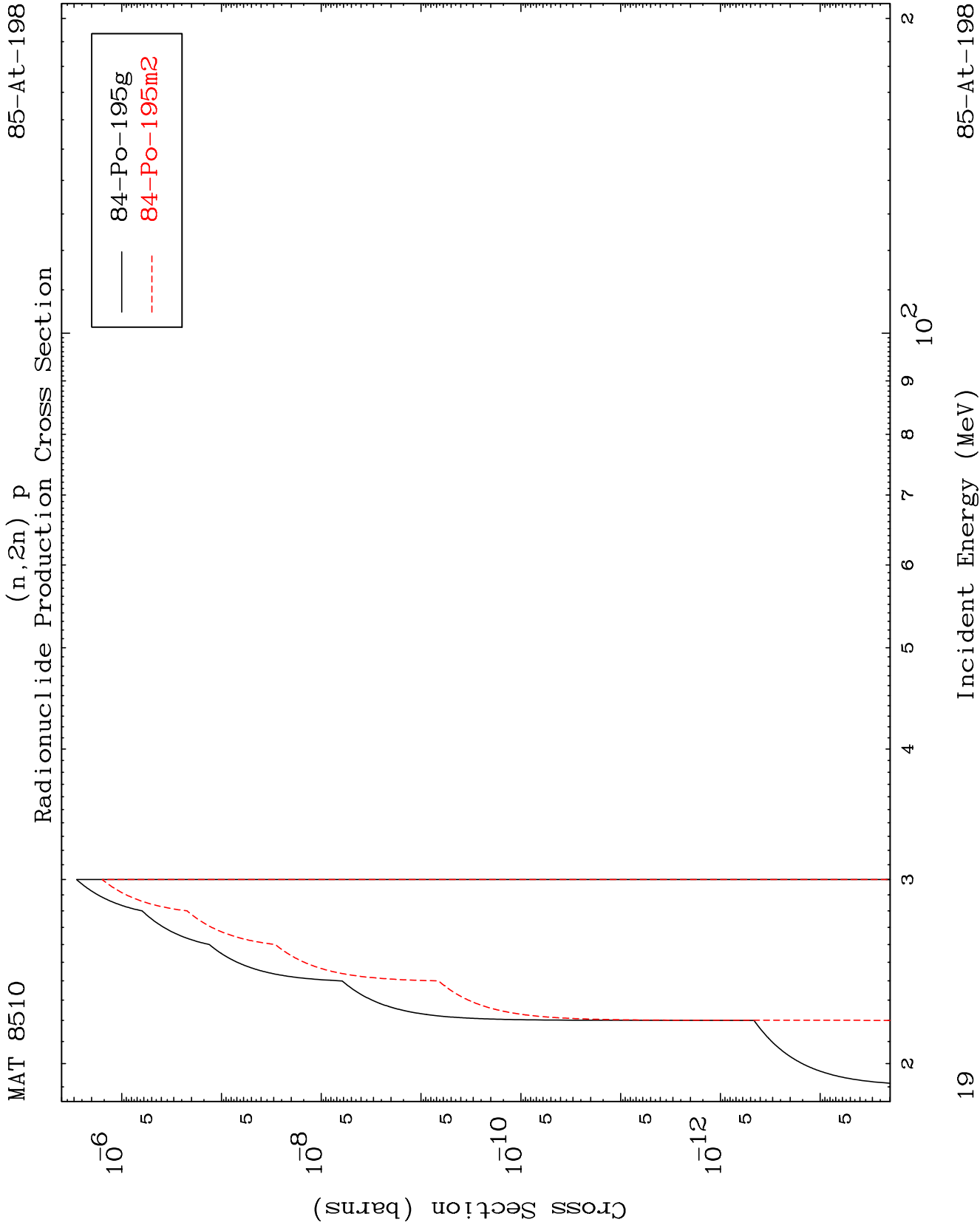
Incident Energy (MeV)

85-At-198

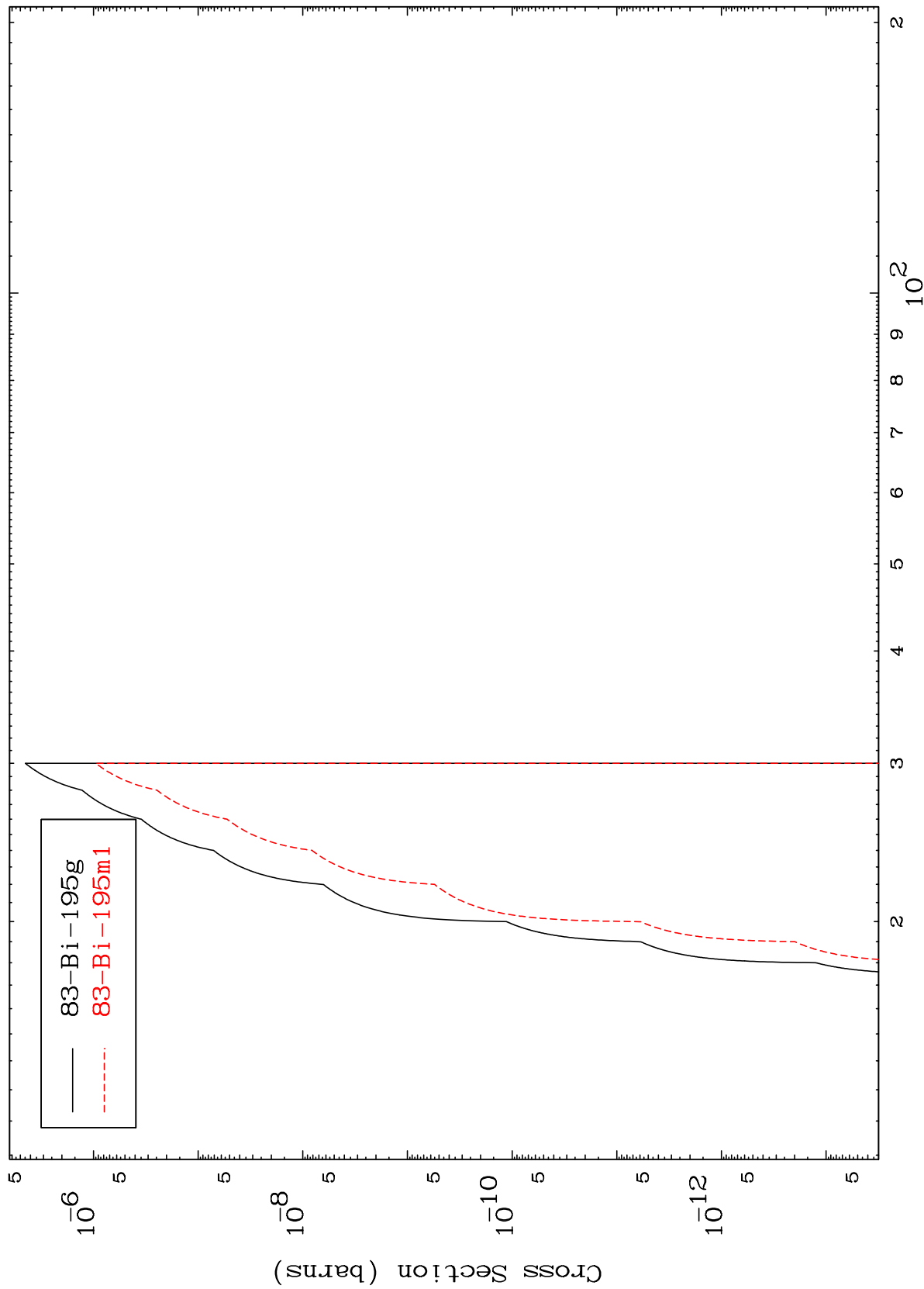


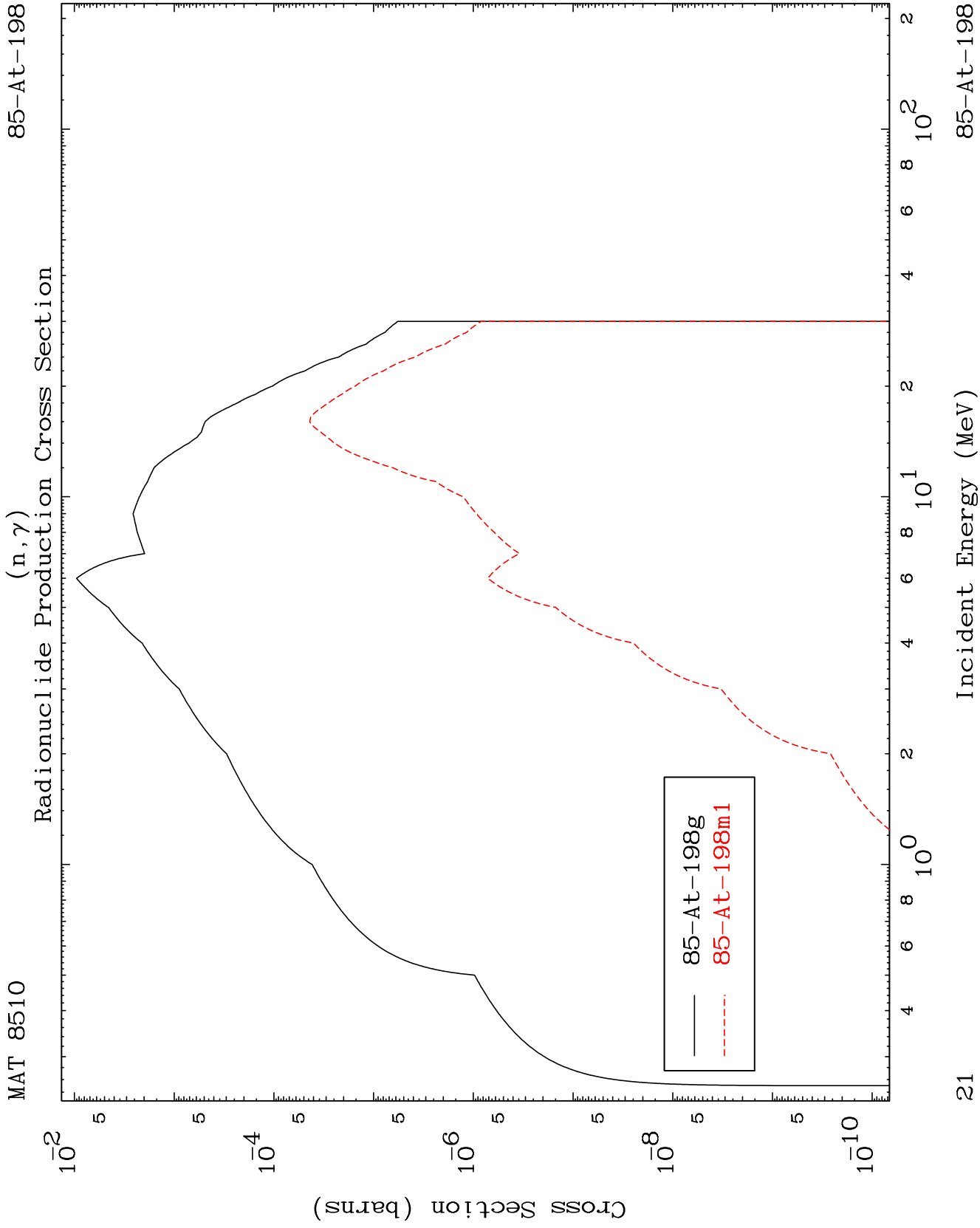
(n,n') d
Radionuclide Production Cross Section





Radionuclide Production Cross Section
(n,2n) p

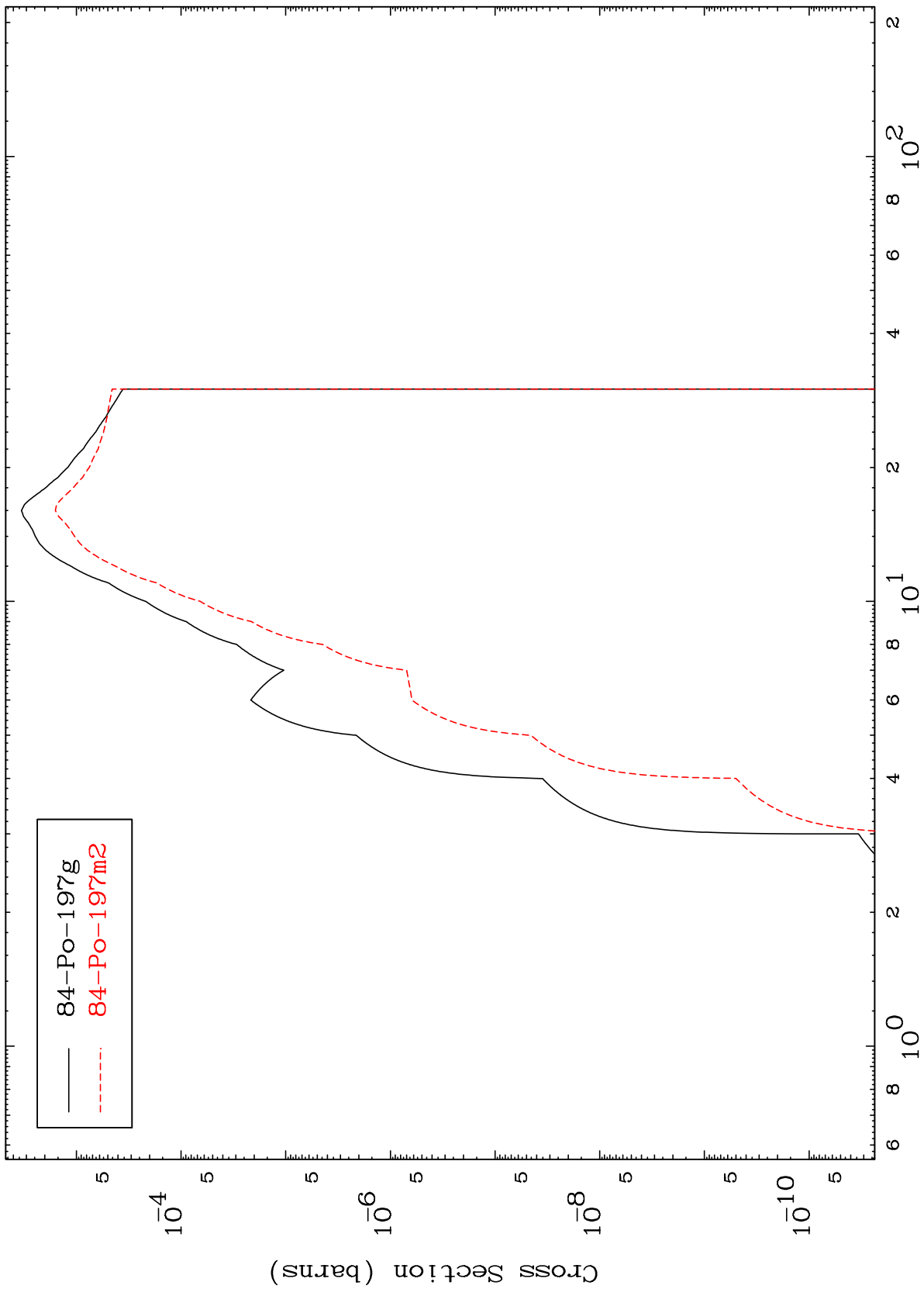




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Radionuclide Production Cross Section (n,p)

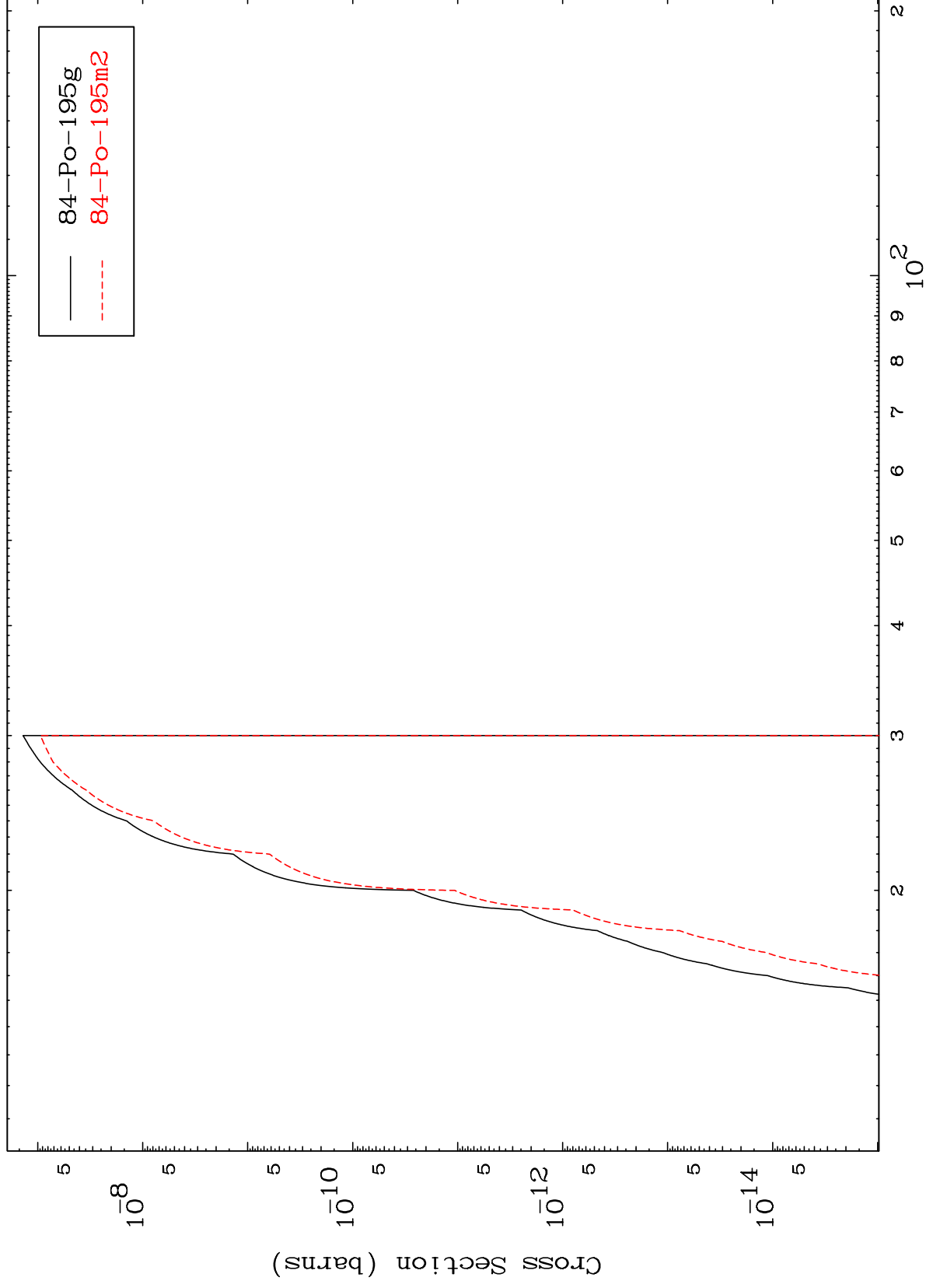


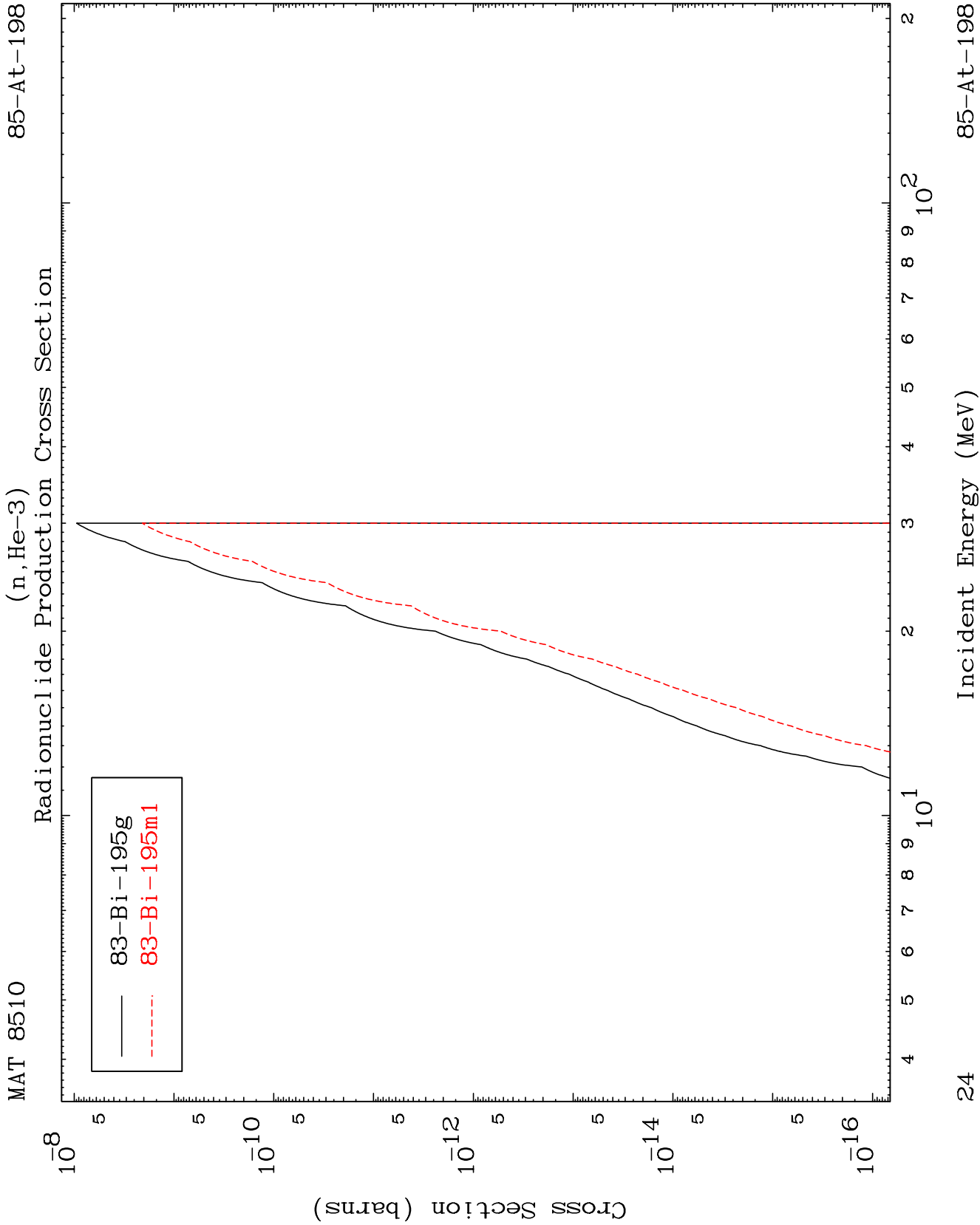
22

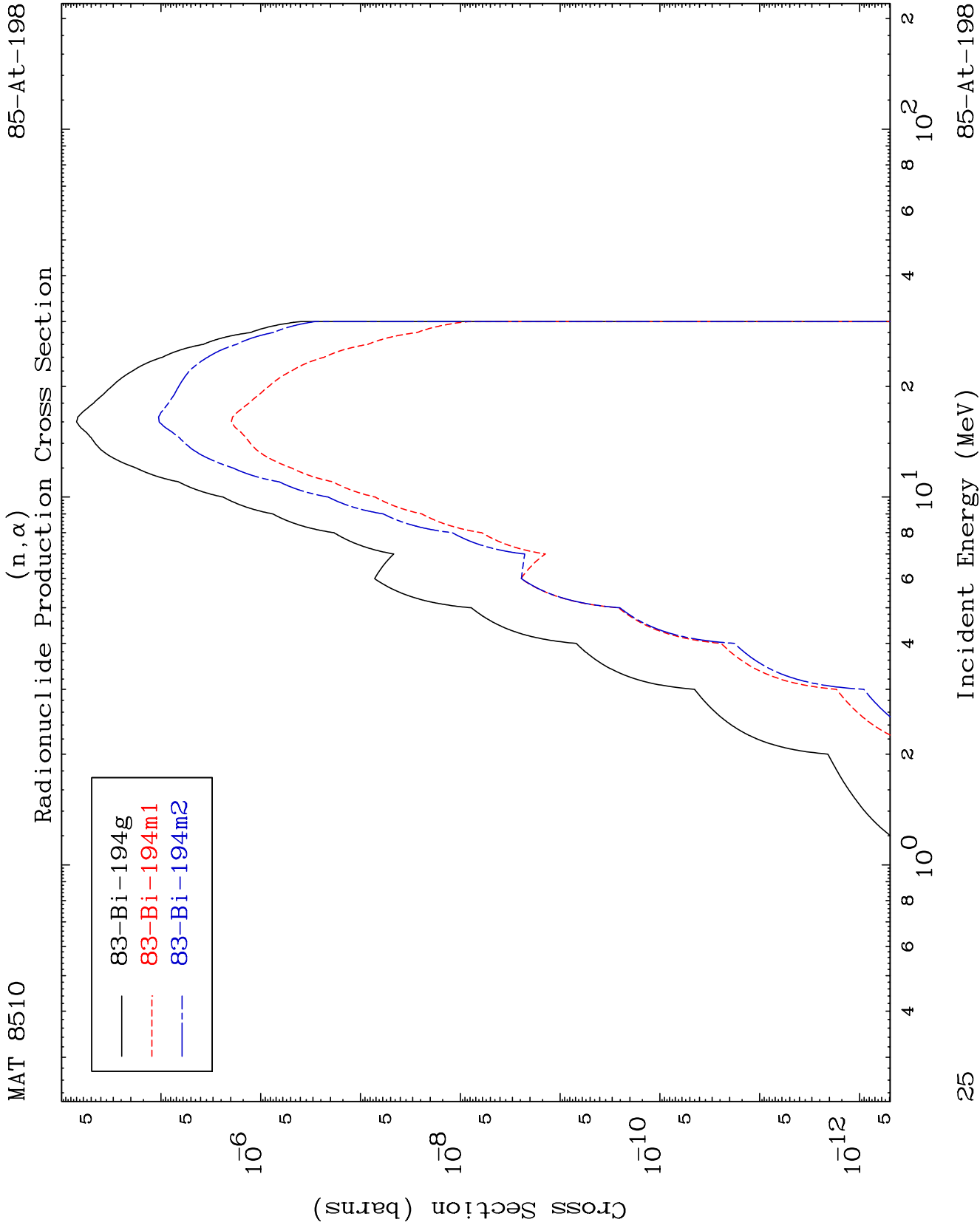
Incident Energy (MeV)

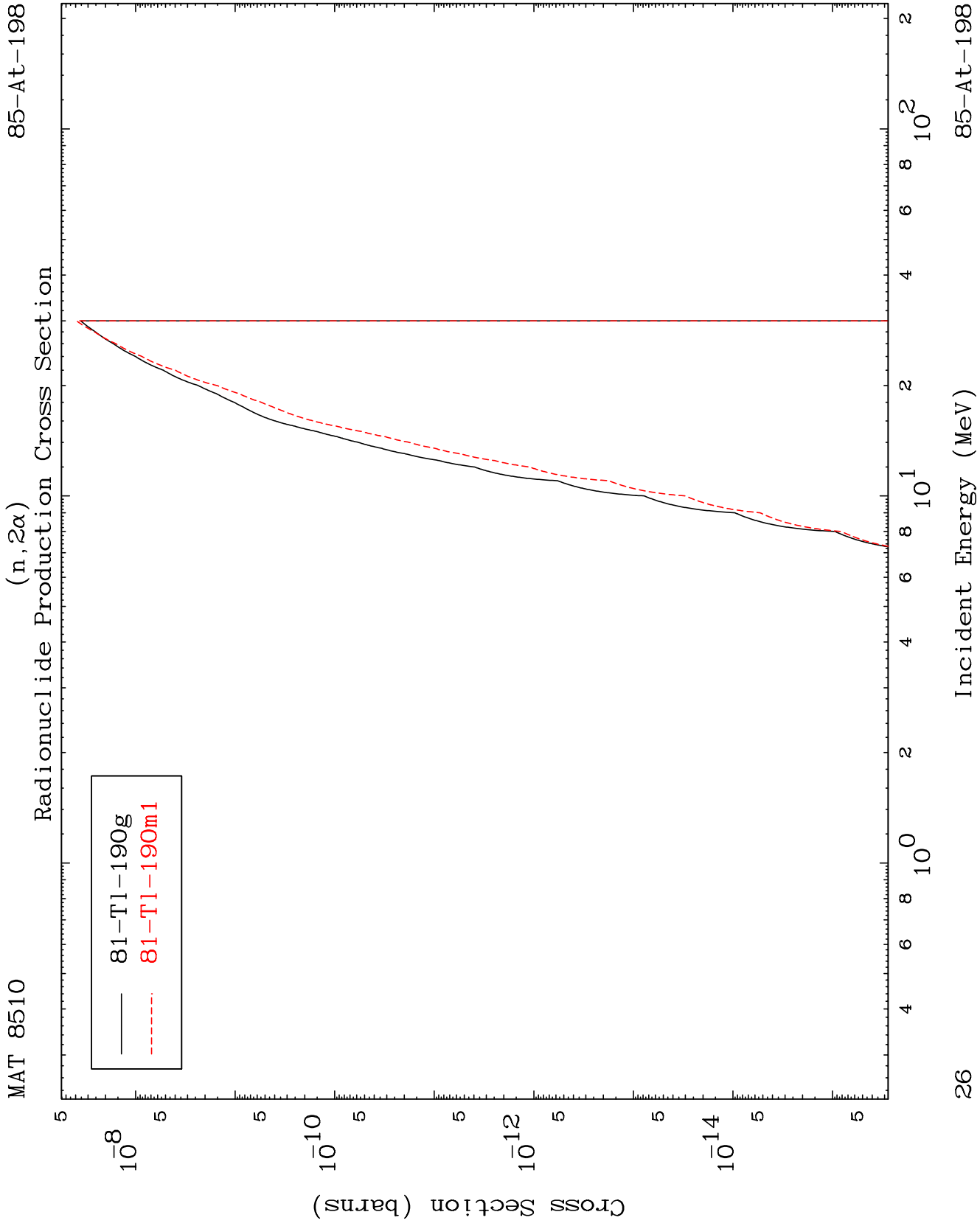
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Radionuclide Production Cross Section





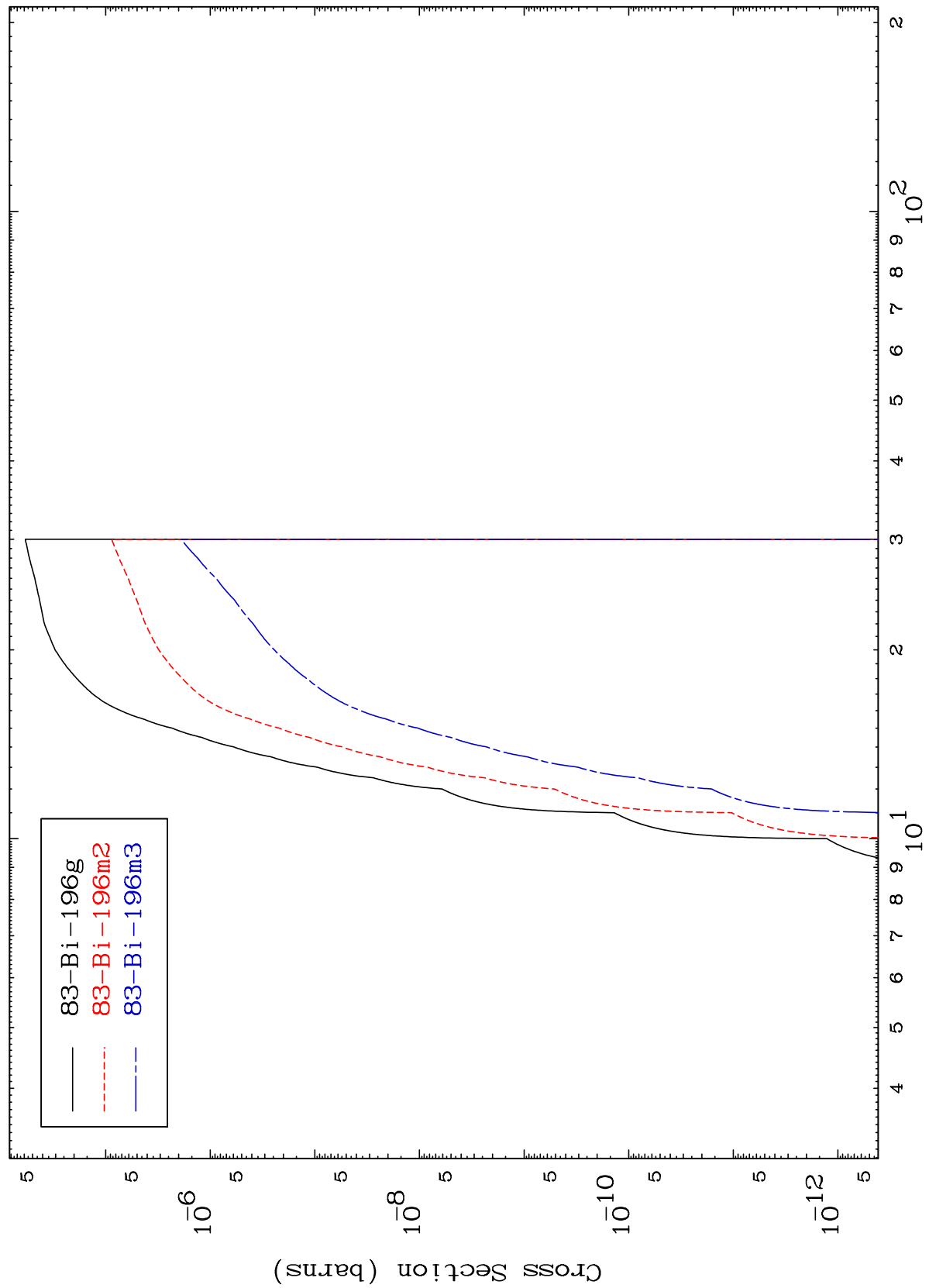




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Radionuclide Production Cross Section
(n,2p)

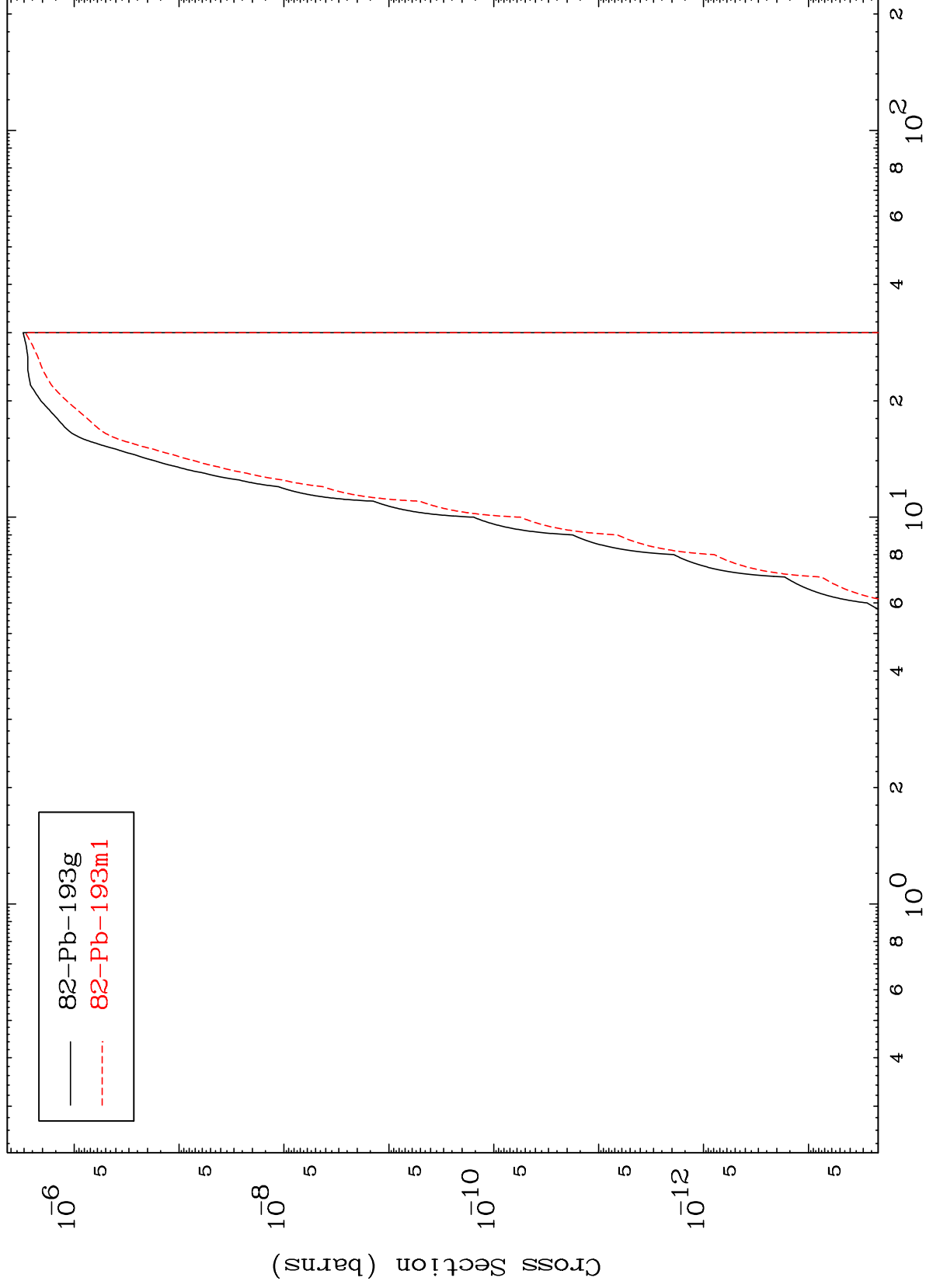


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Incident Energy (MeV)

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(n,p) α
Radionuclide Production Cross Section

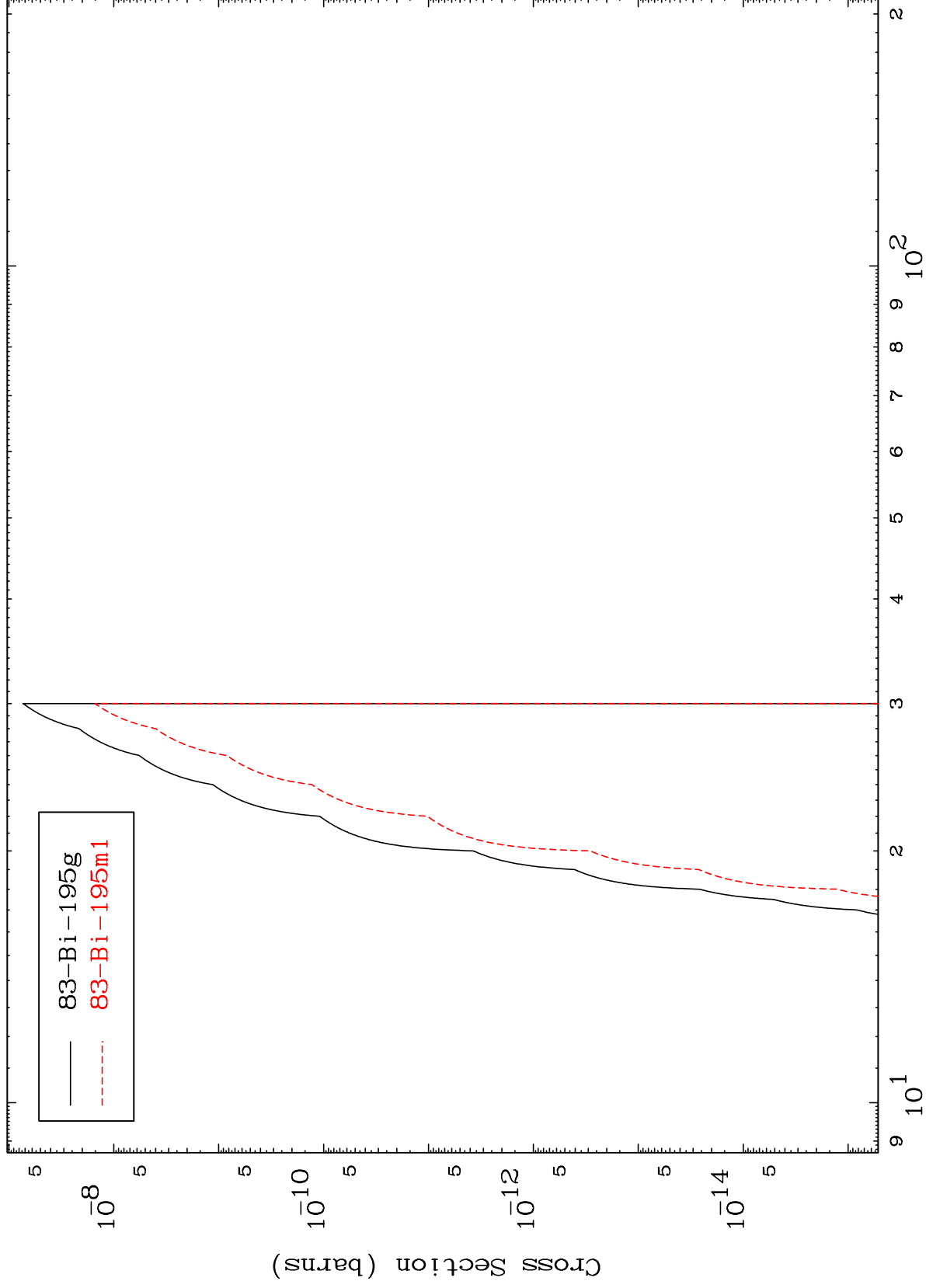


MAT 8510

(n,p) d

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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