

Program EVALPLOT
(Version 2021-1)

by

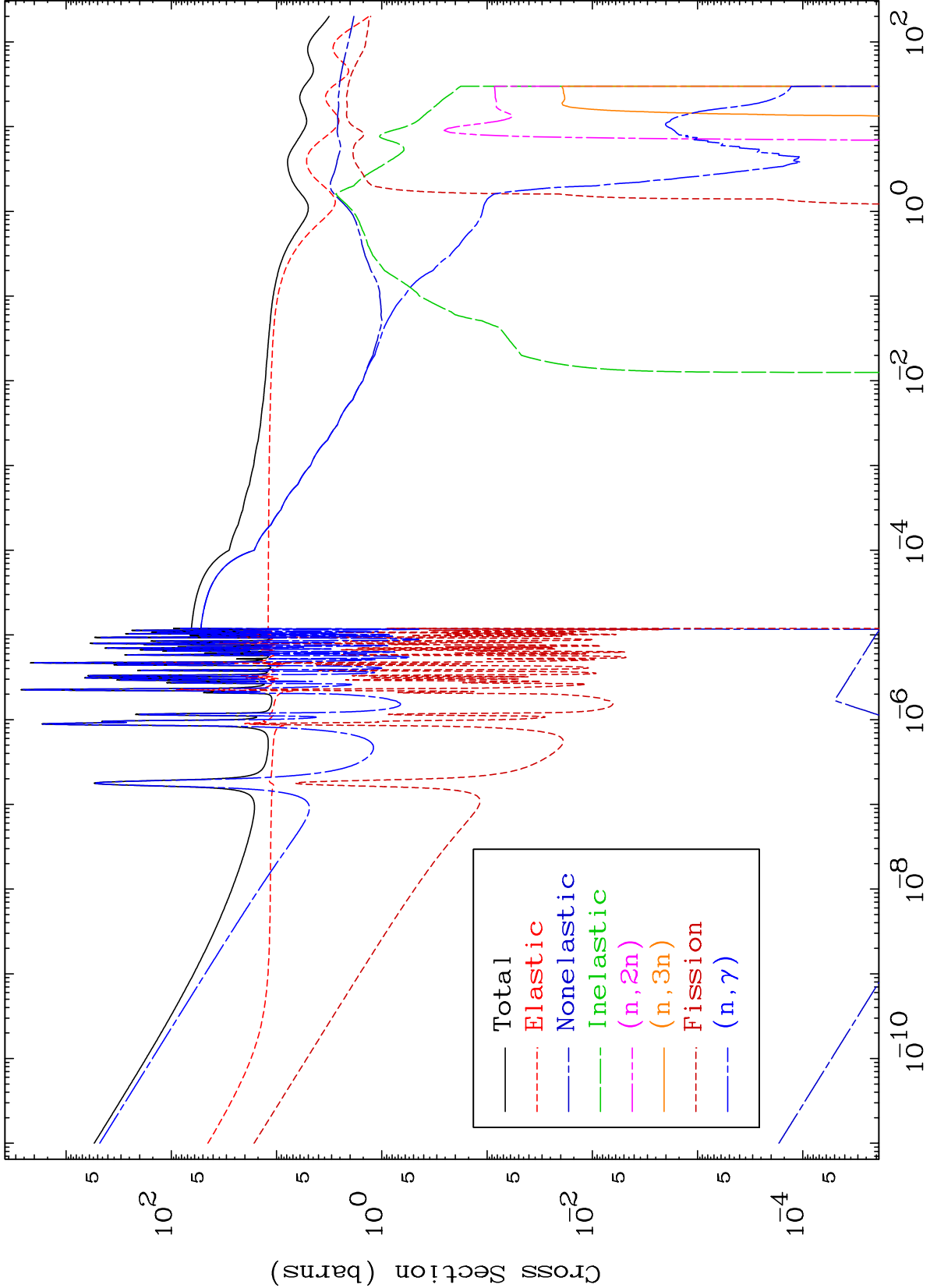
Dermott E. Cullen
(Present Contact Information)

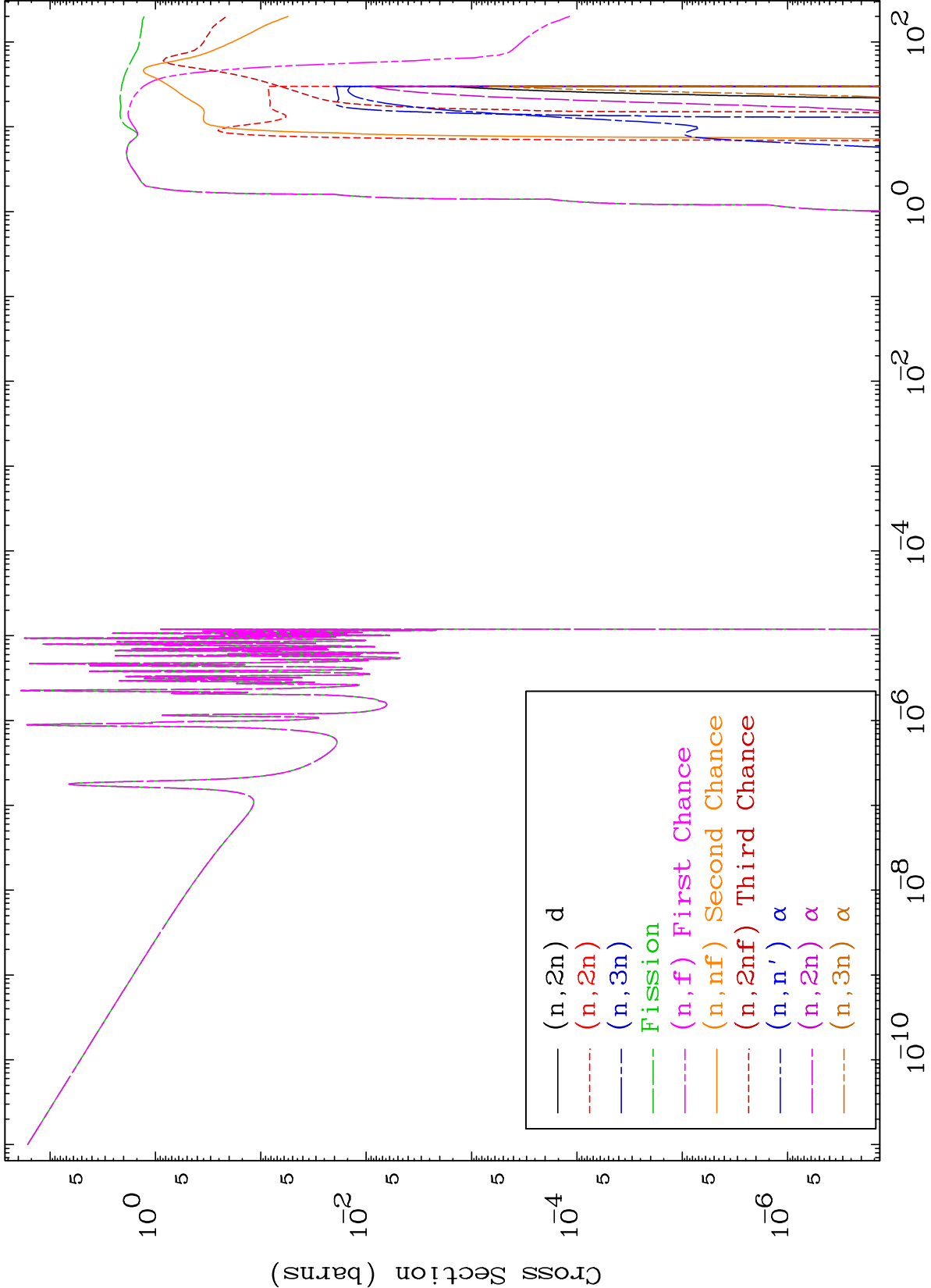
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

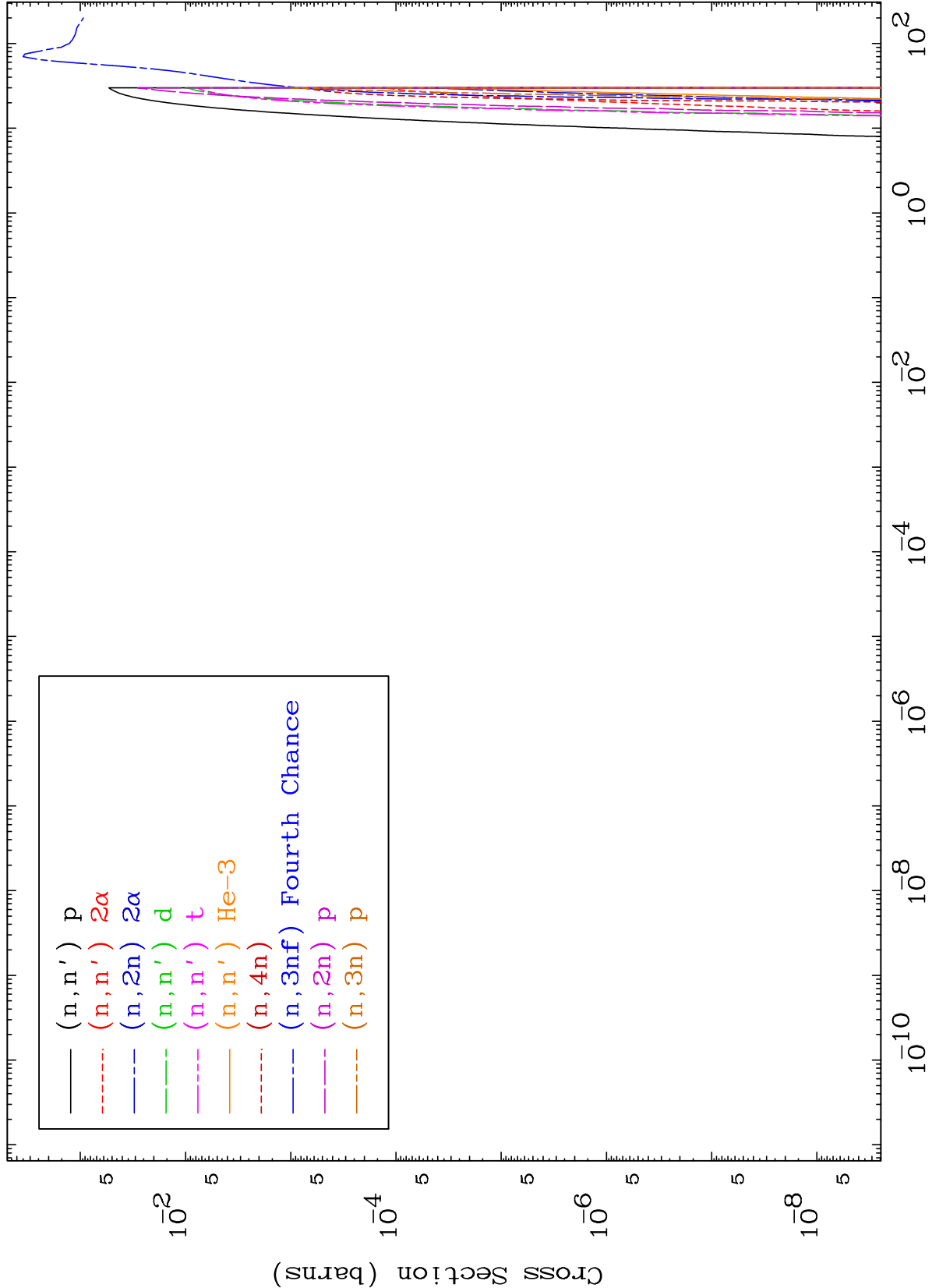
Tele: 925-443-1911

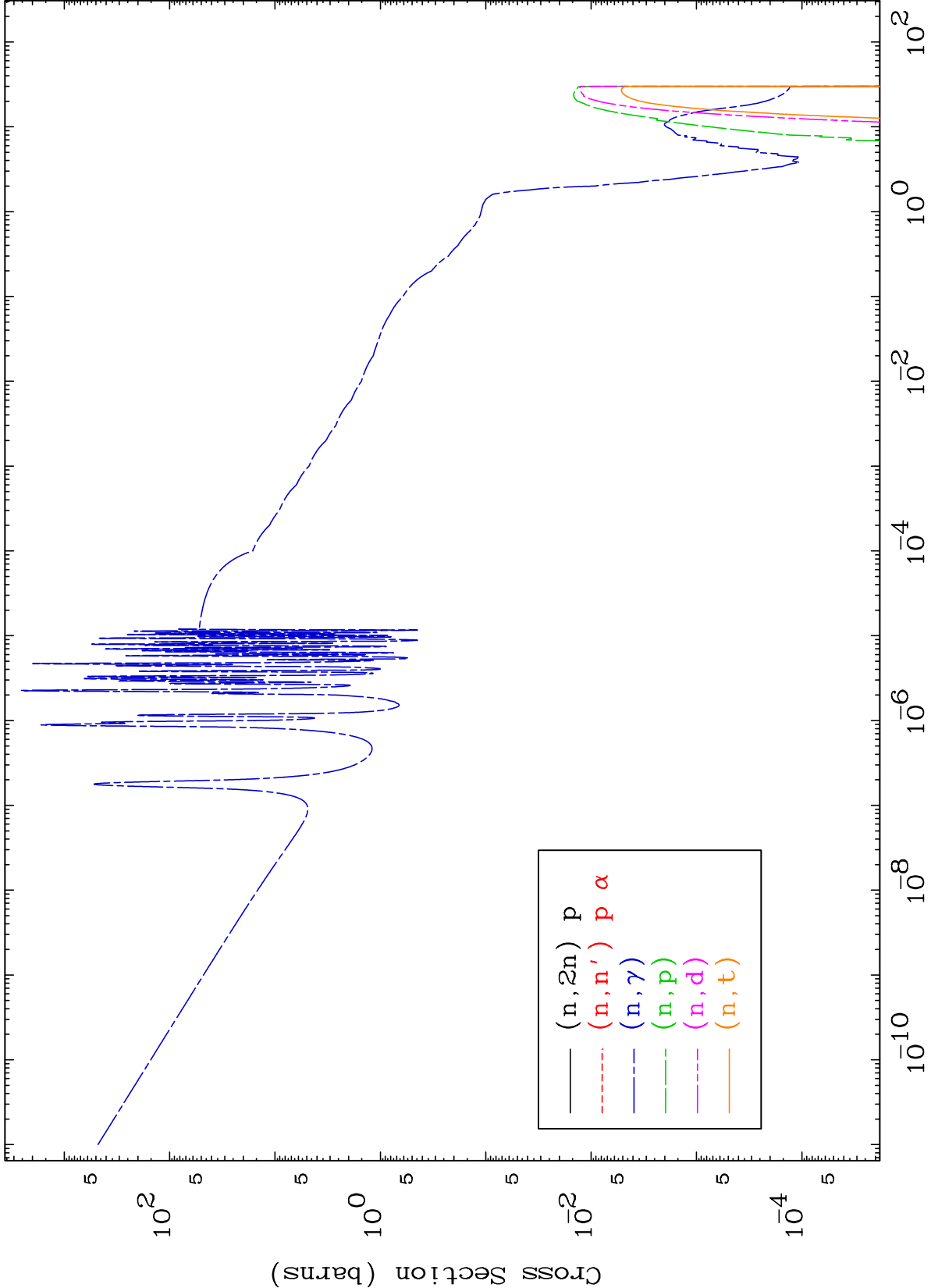
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start





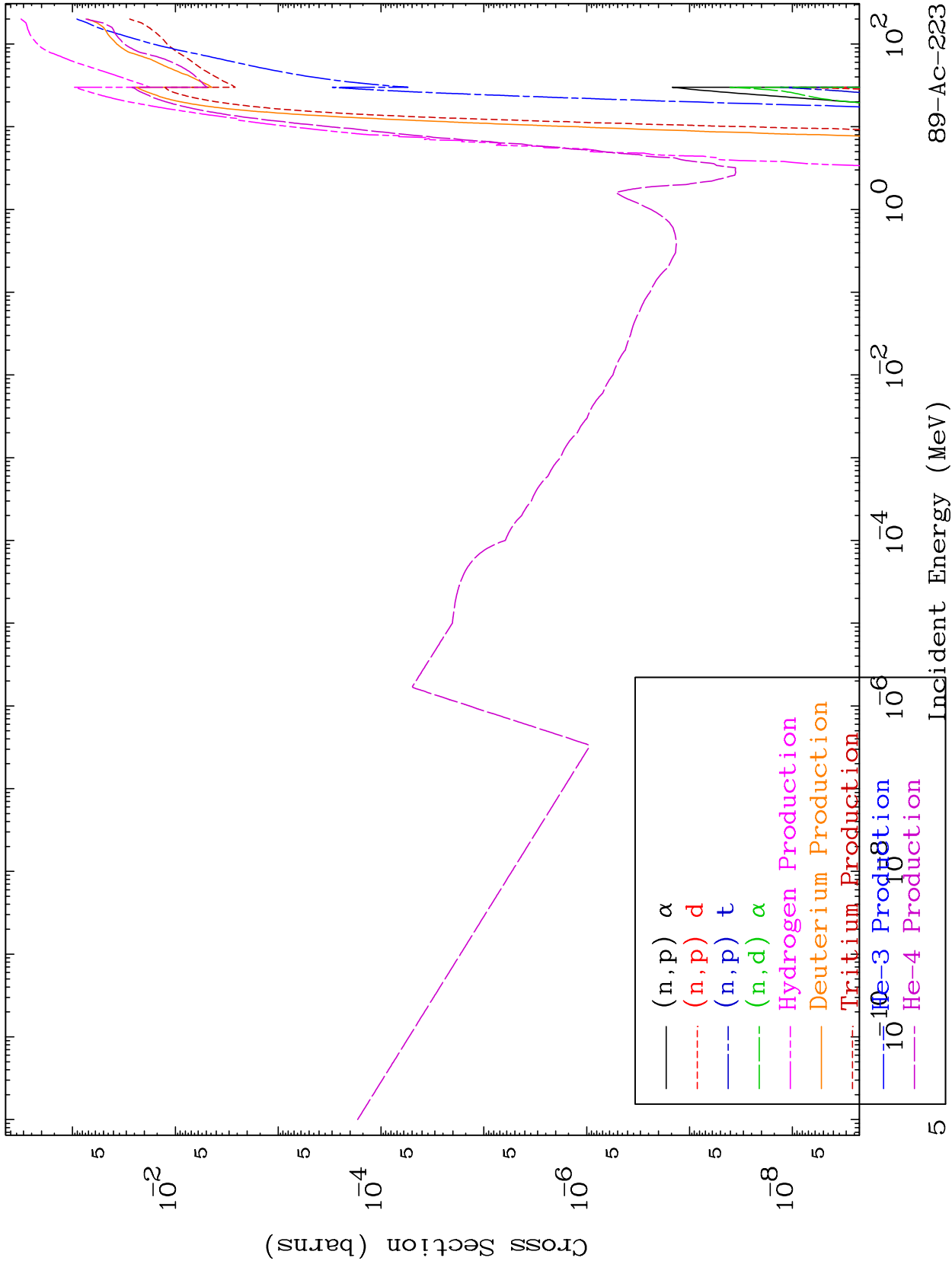


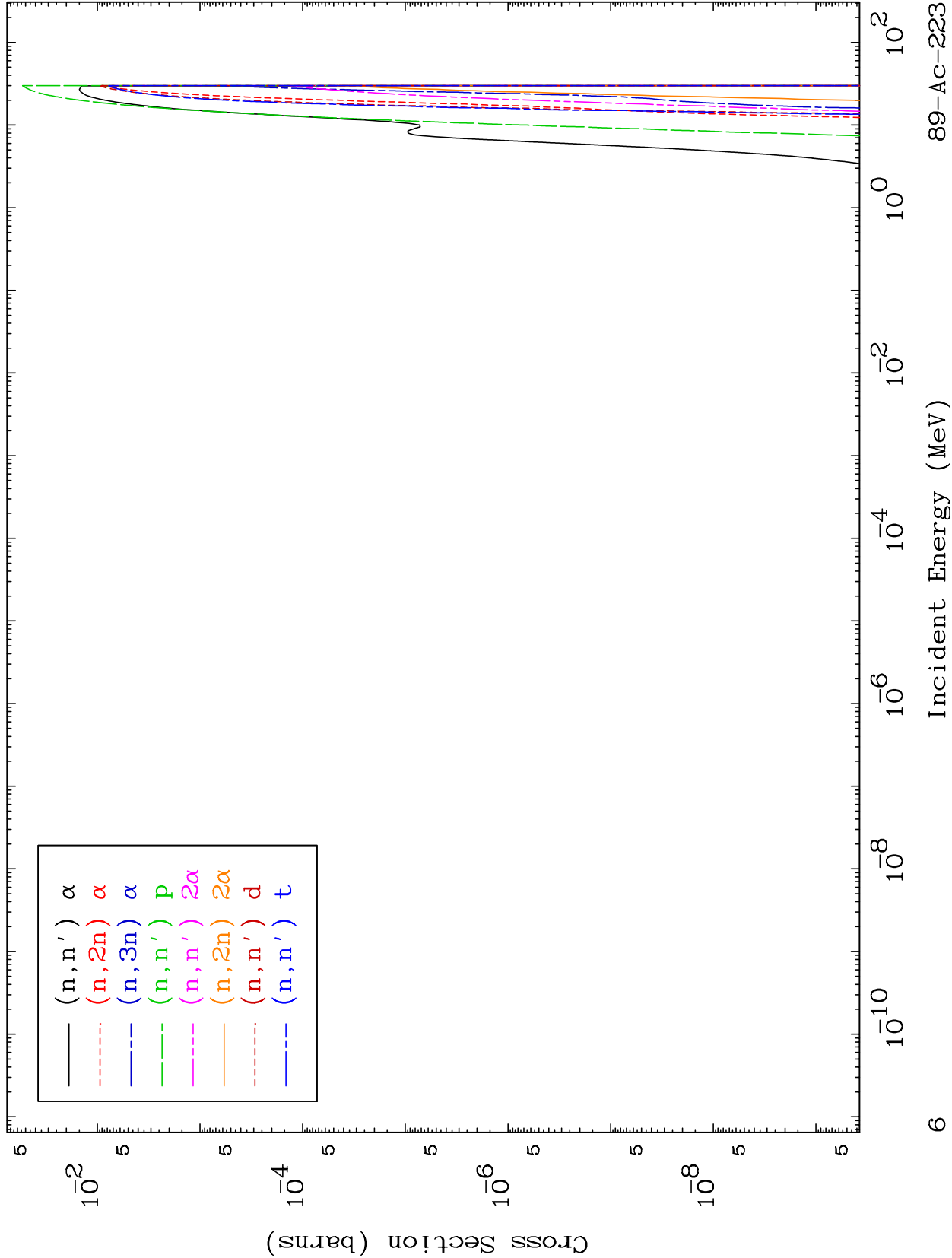


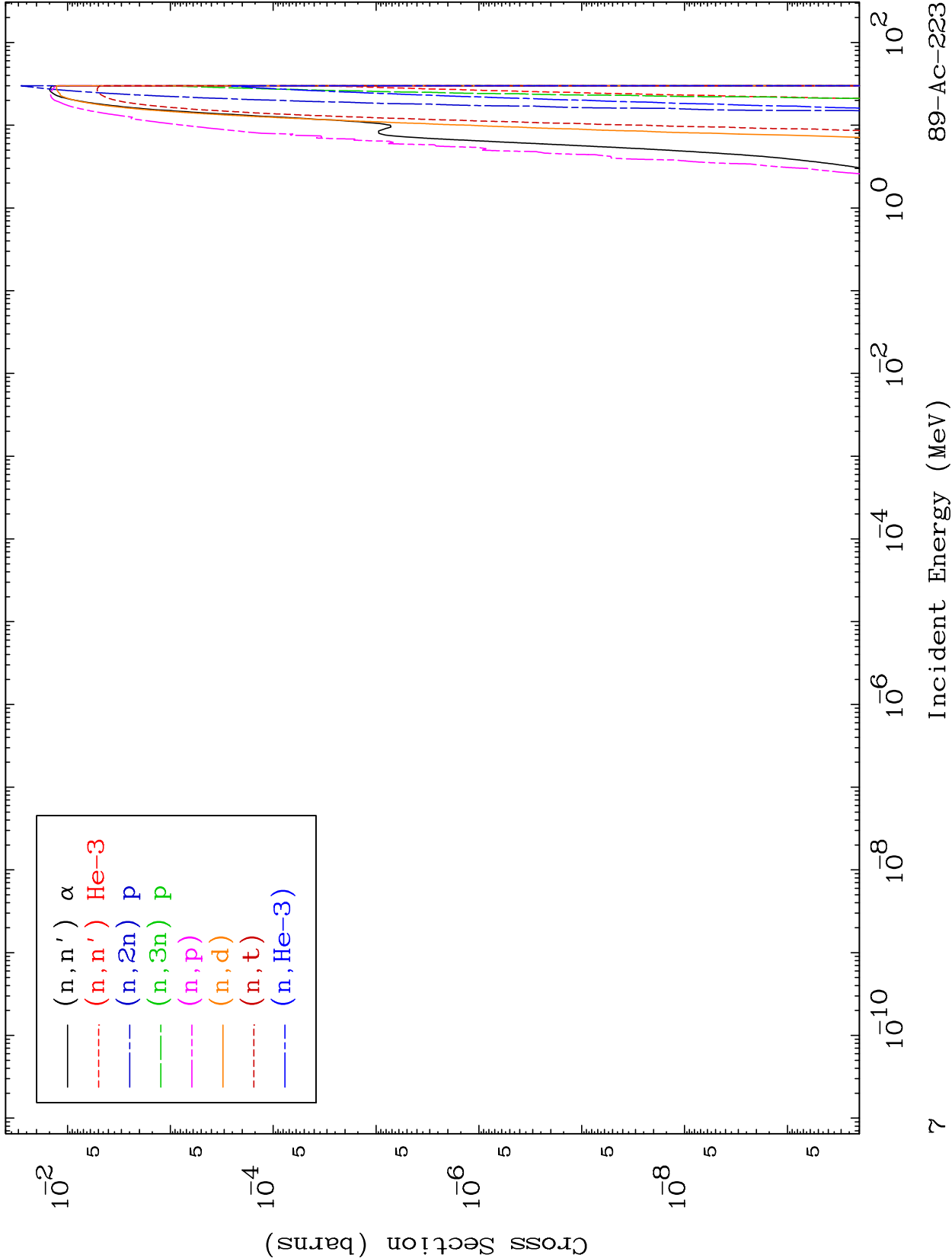
MAT 8919

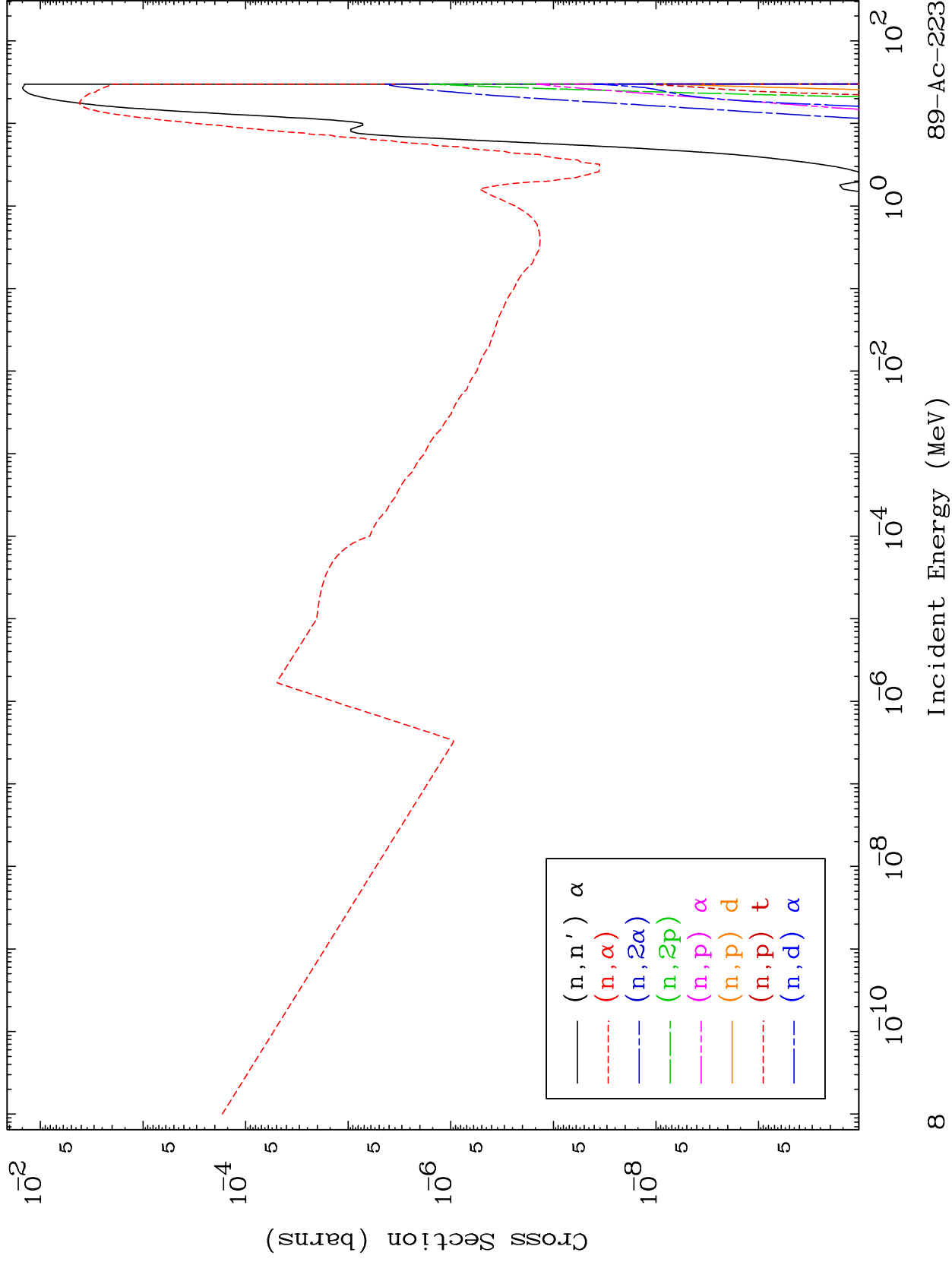
Neutron Absorption
293 Kelvin Cross Sections

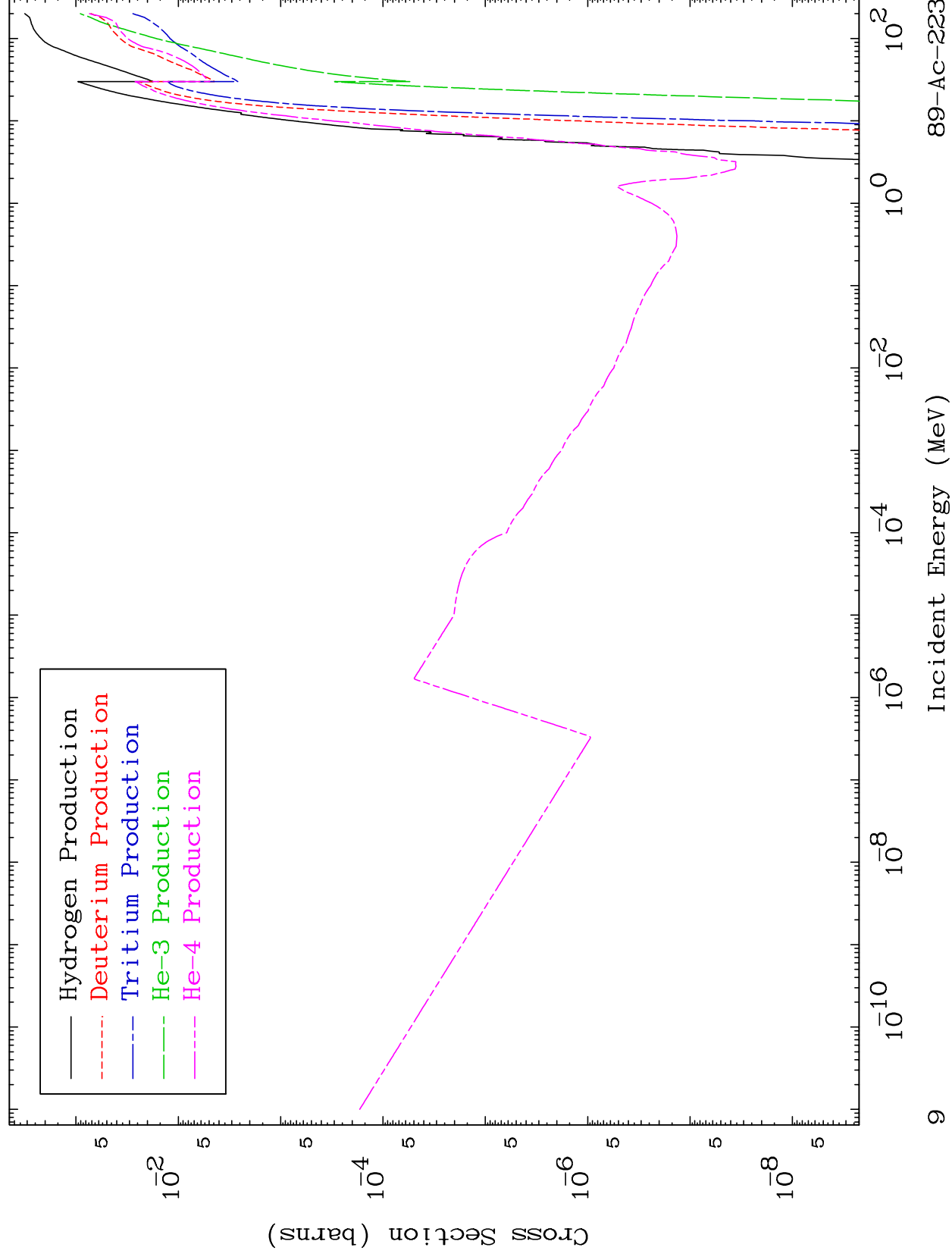
89-Ac-223









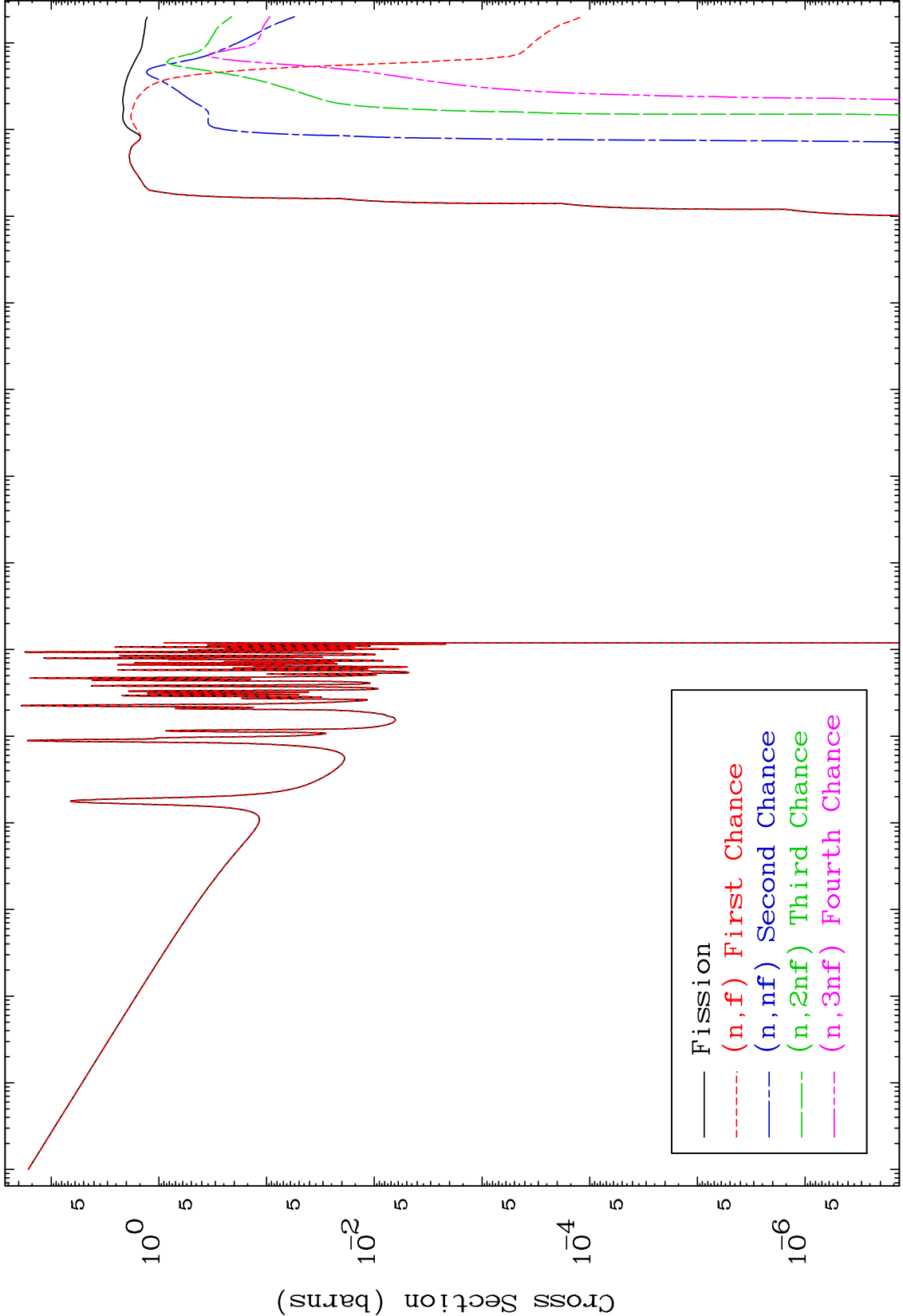


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Fission

293 Kelvin Cross Sections

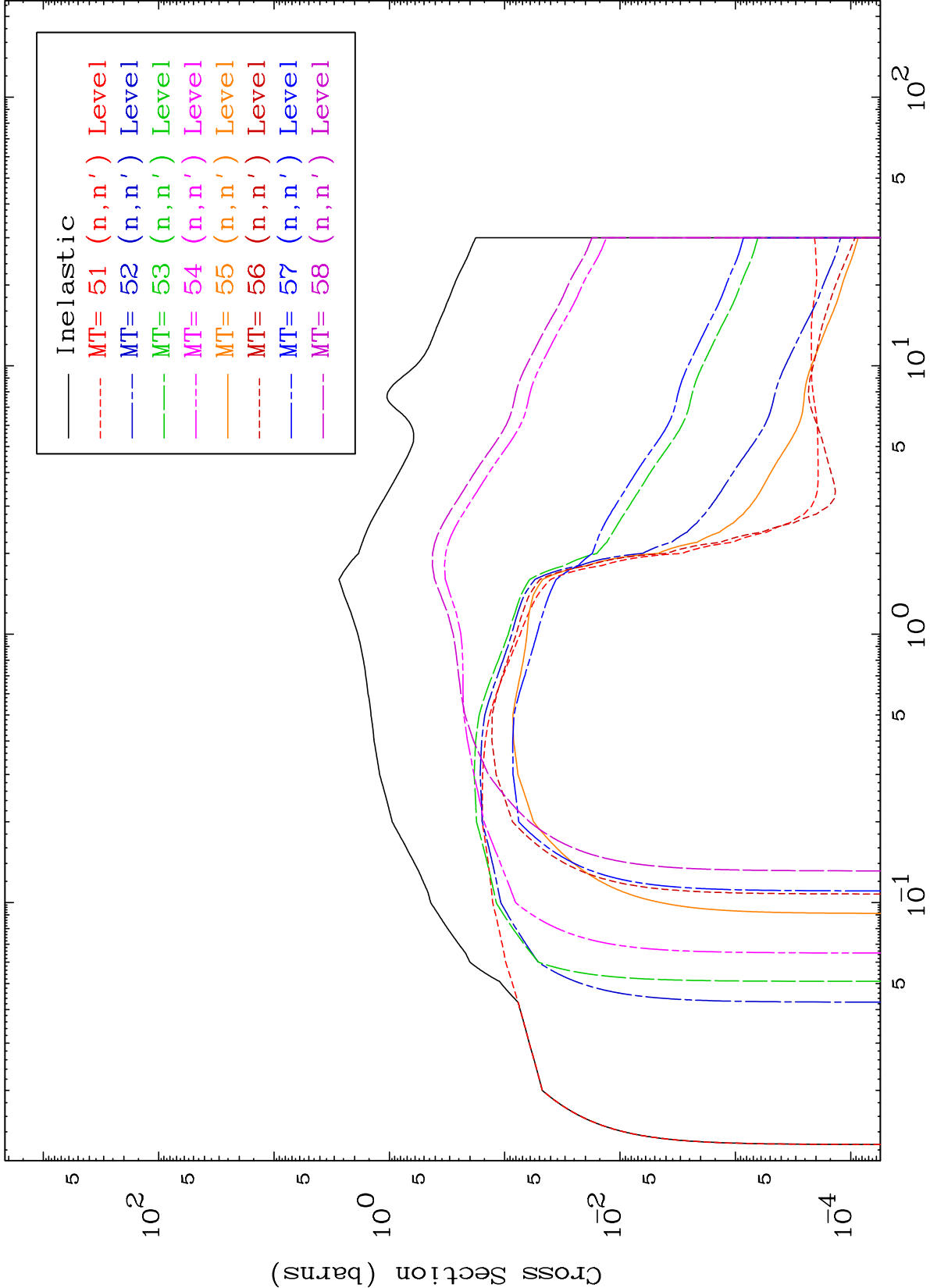
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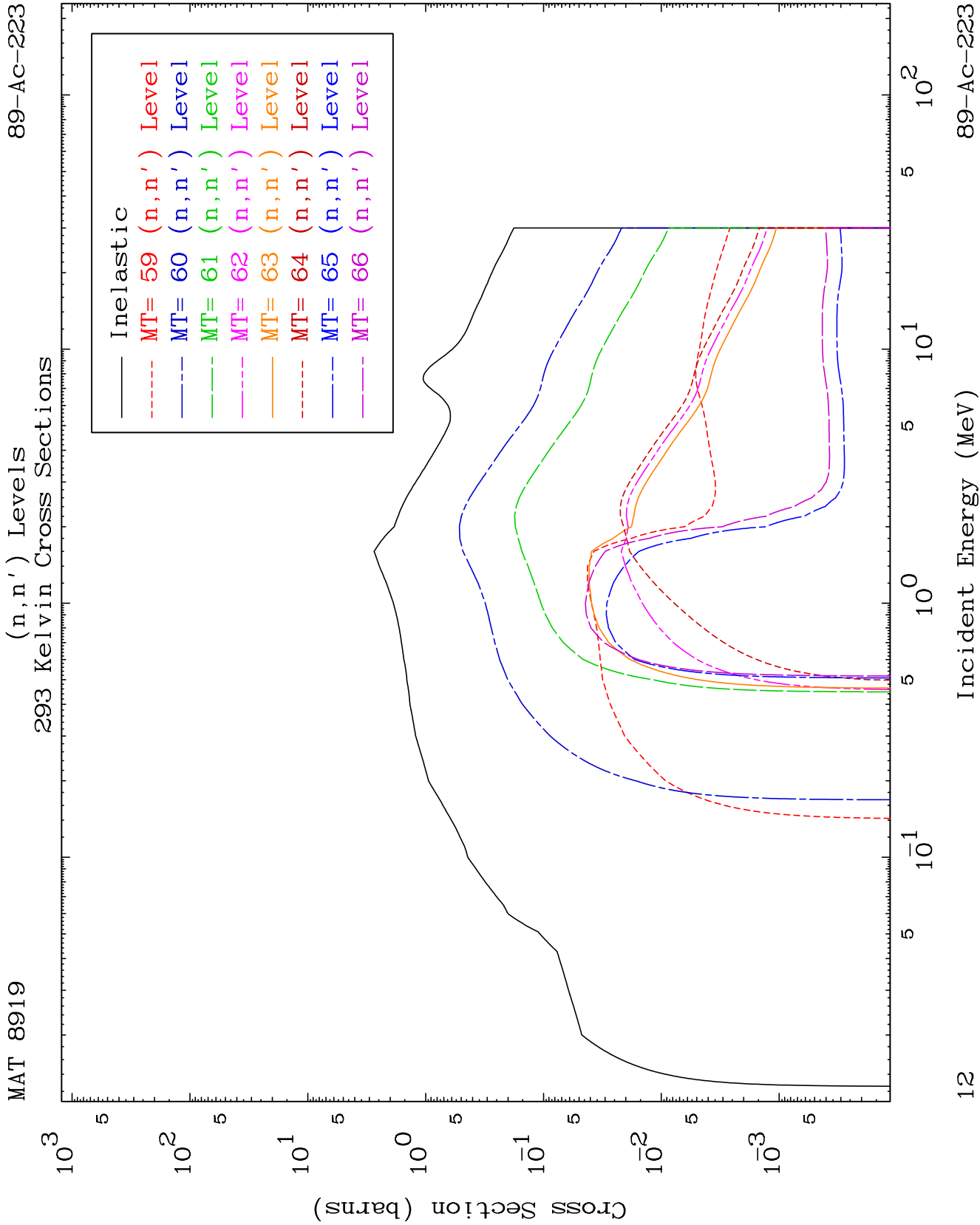


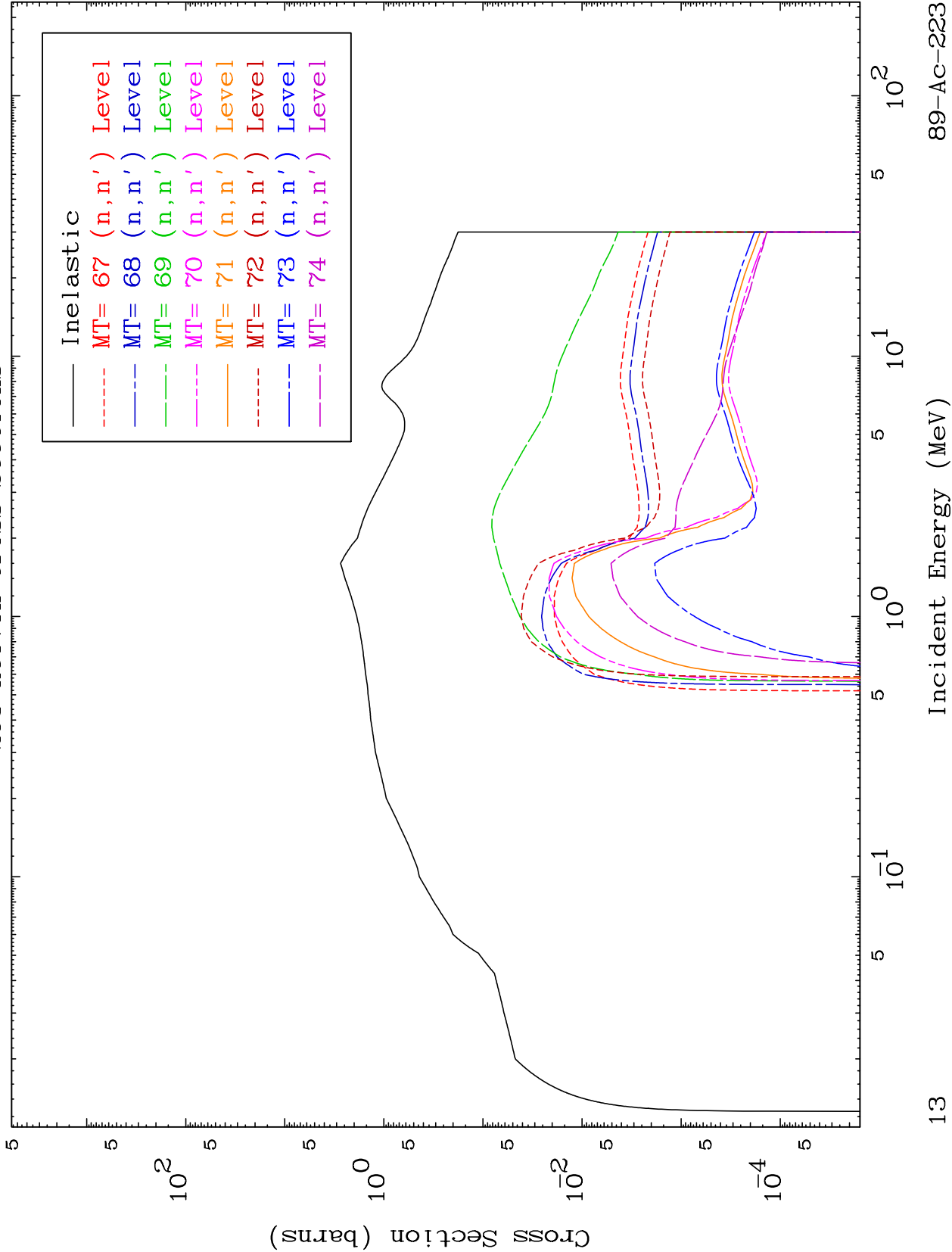
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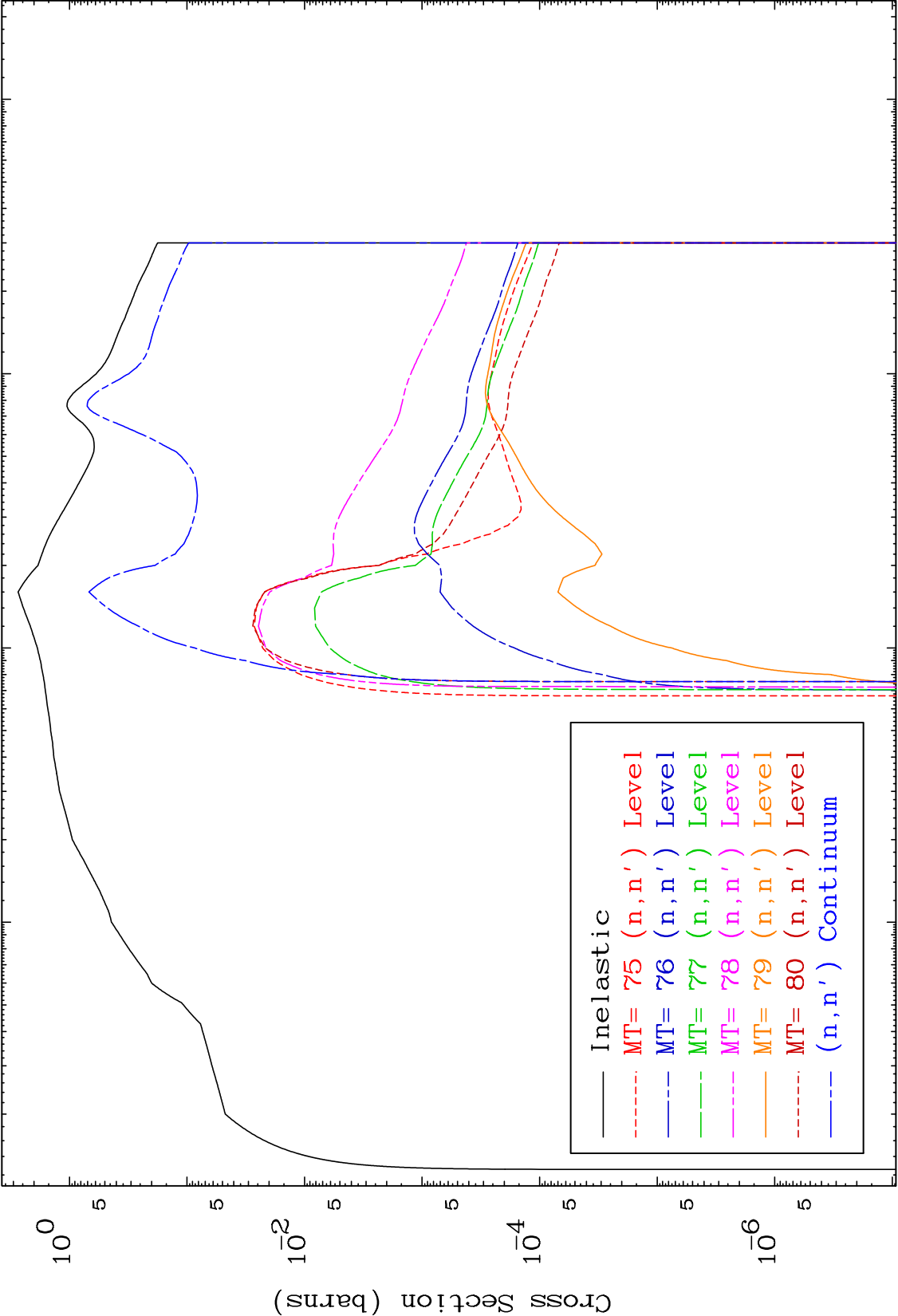
Incident Energy (MeV)

89-Ac-223





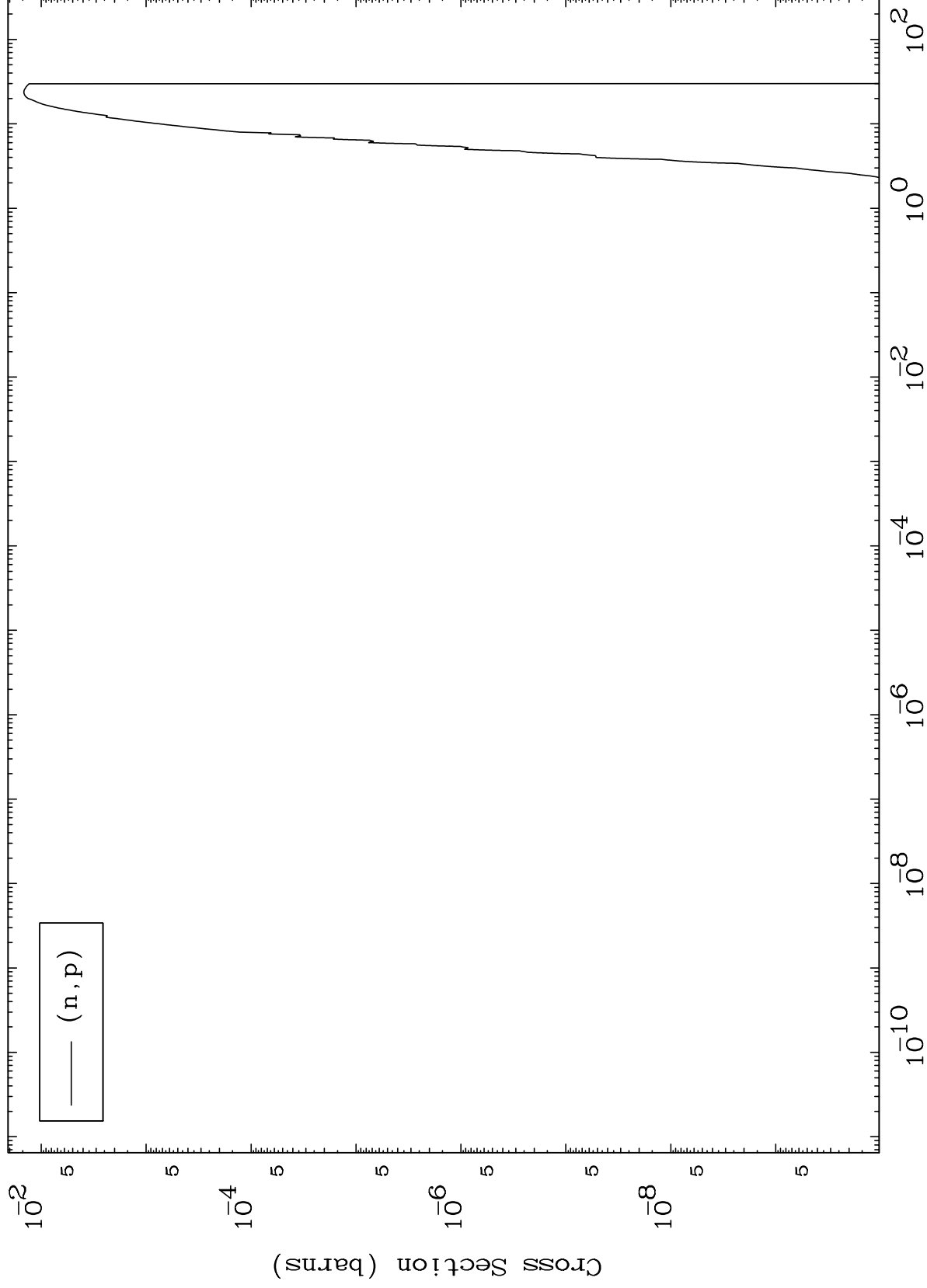




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(n,p) Levels
293 Kelvin Cross Sections



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Incident Energy (MeV)

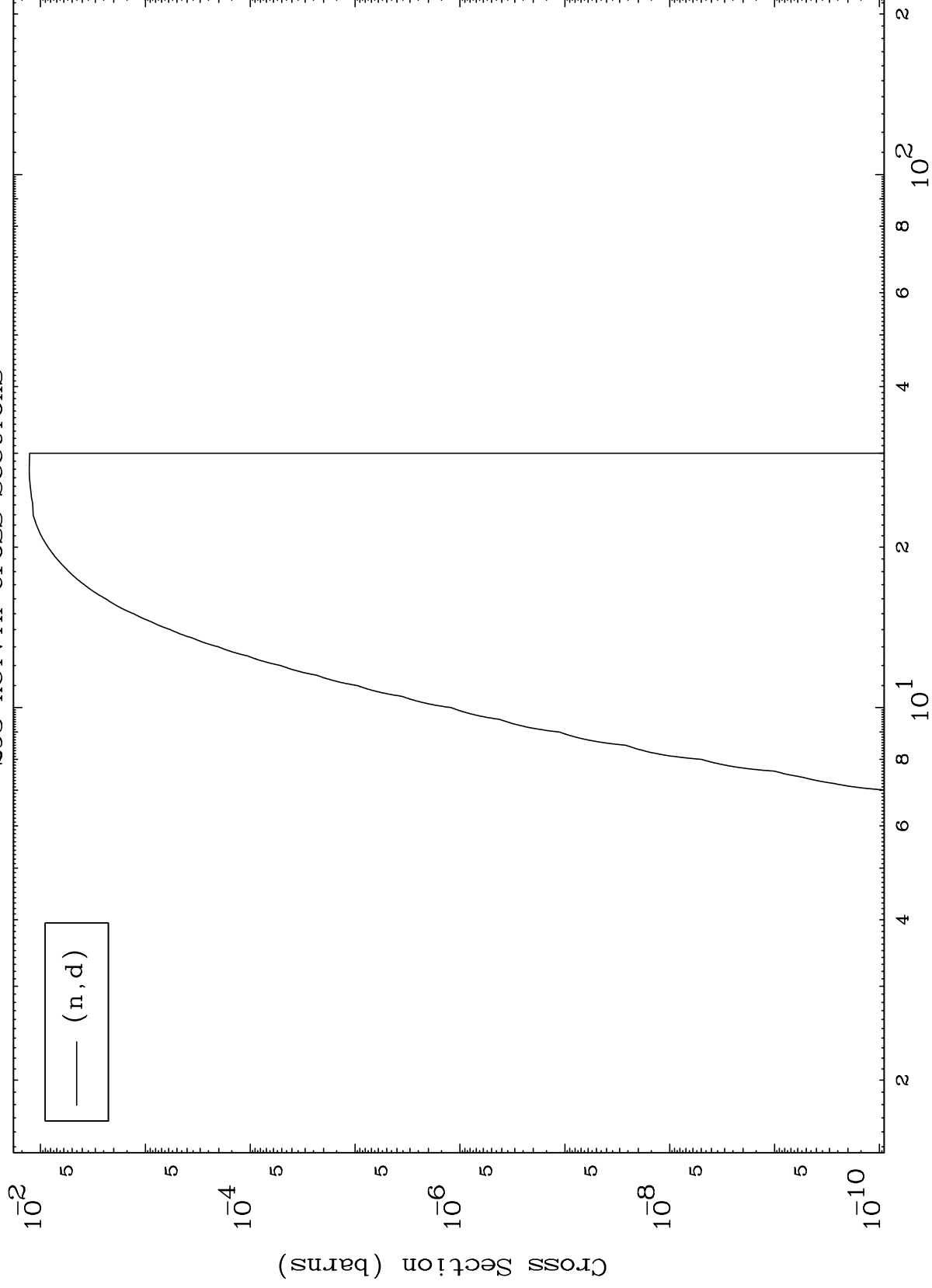
15

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(n,d) Levels

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293 Kelvin Cross Sections



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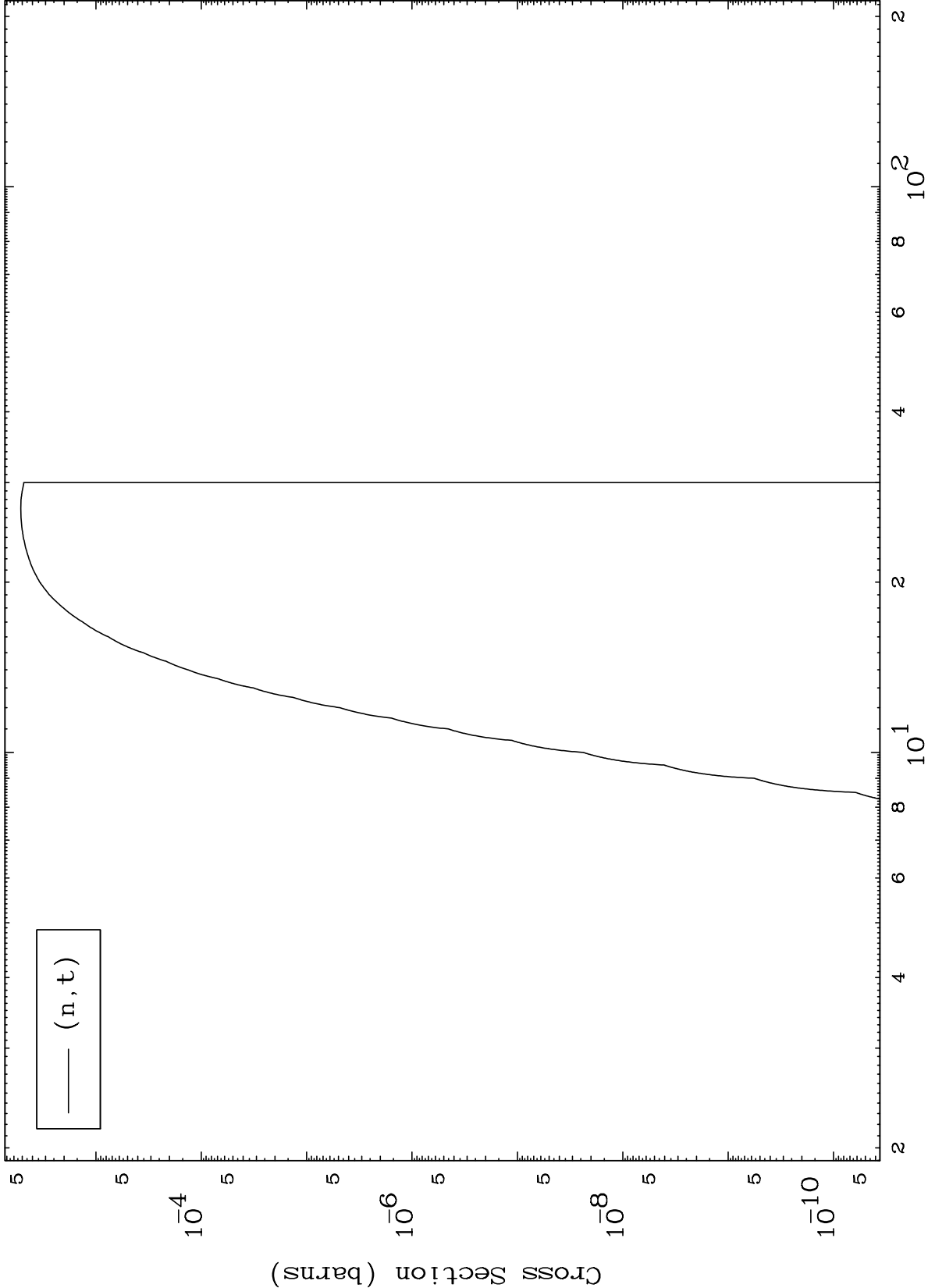
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

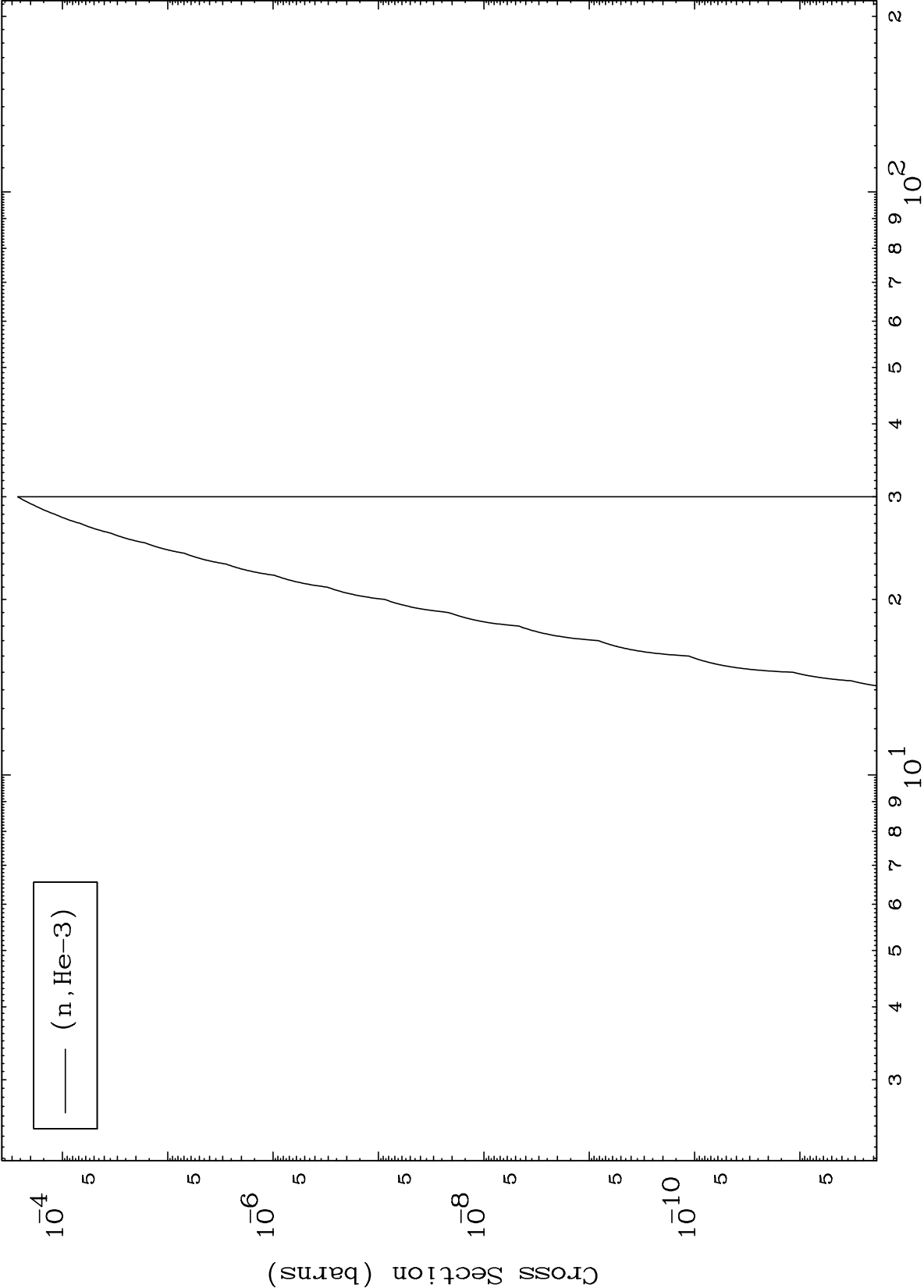
89-Ac-223



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Incident Energy (MeV)

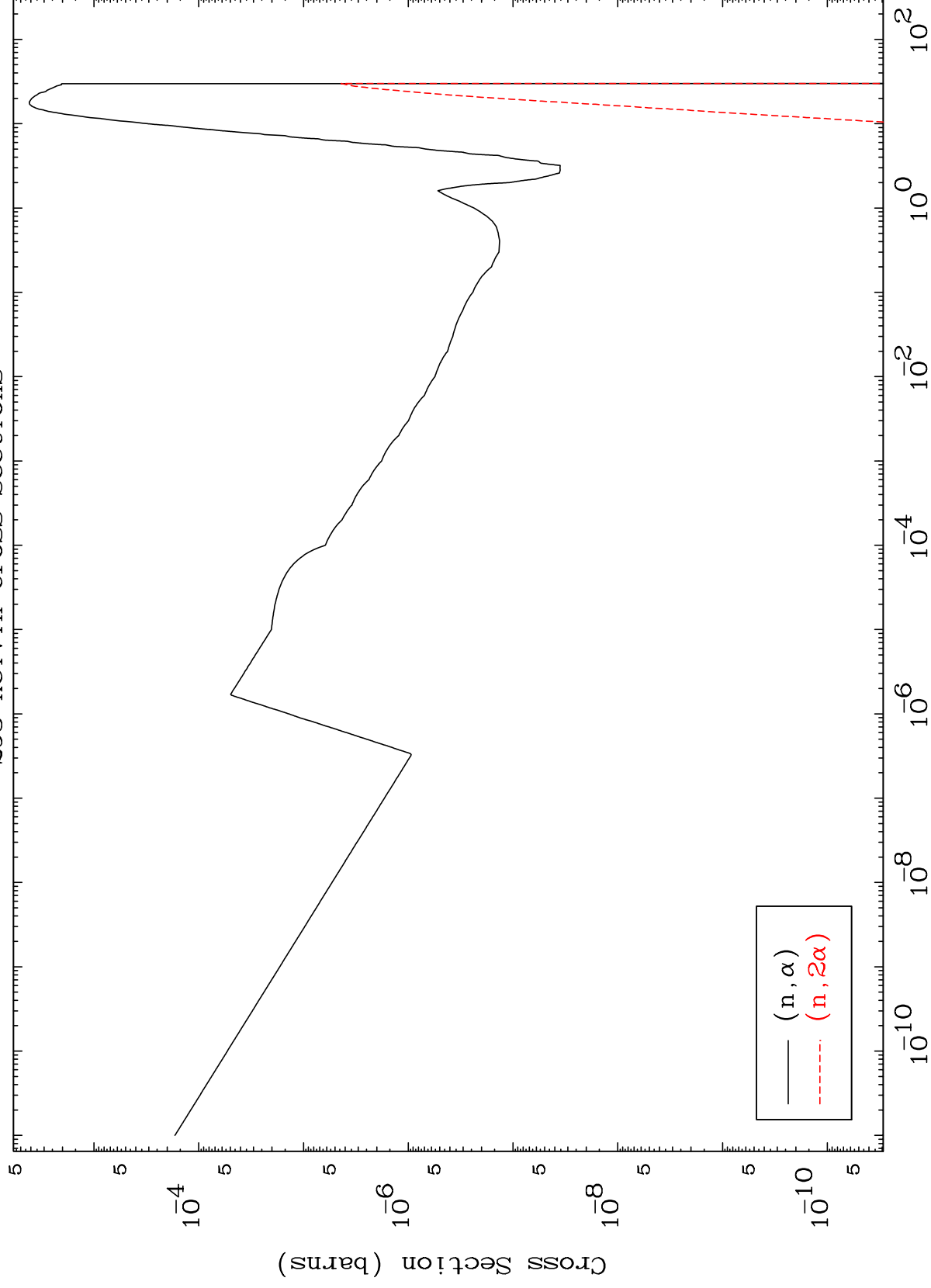
89-Ac-223



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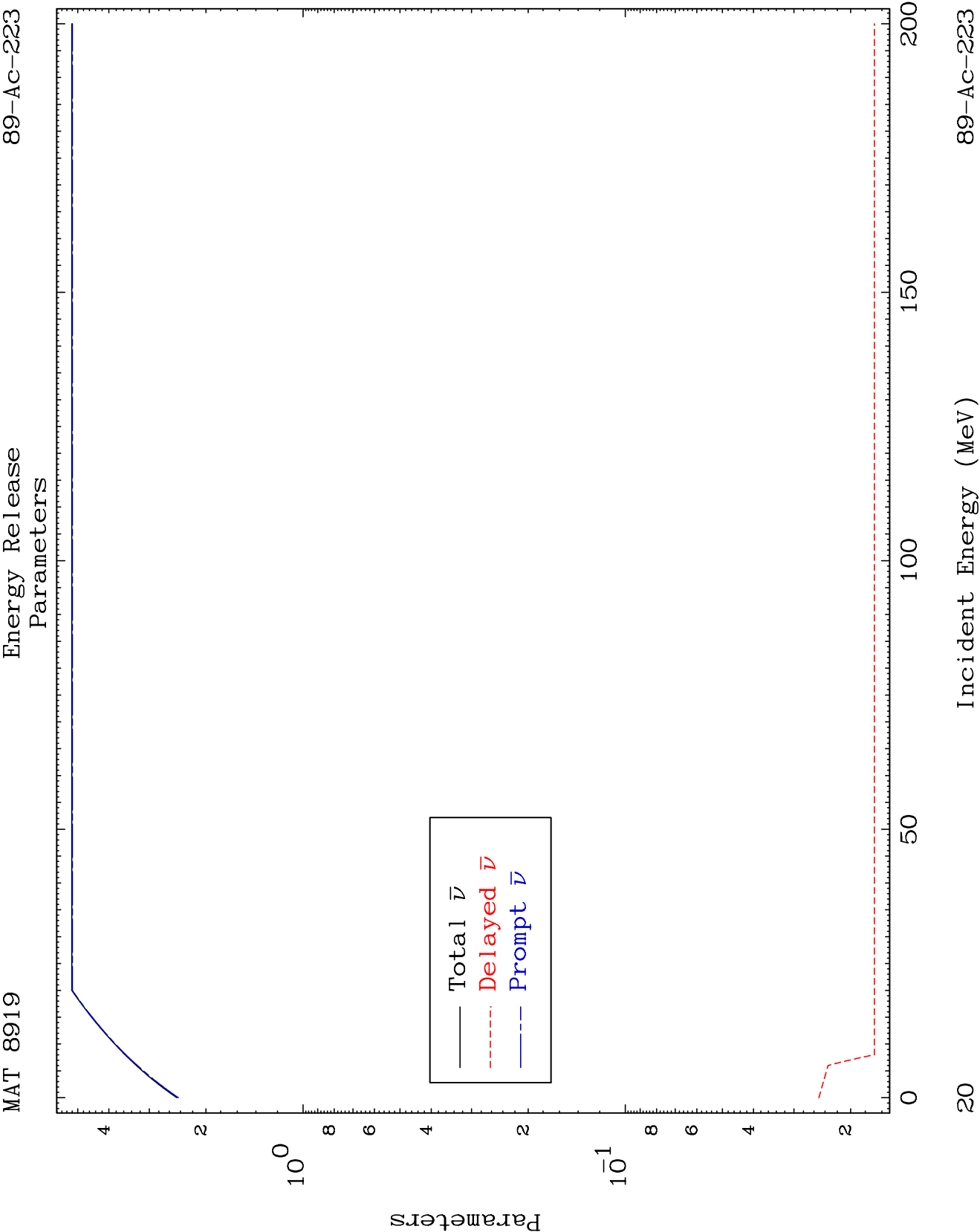
(n, α) Levels
293 Kelvin Cross Sections

89-Ac-223



19

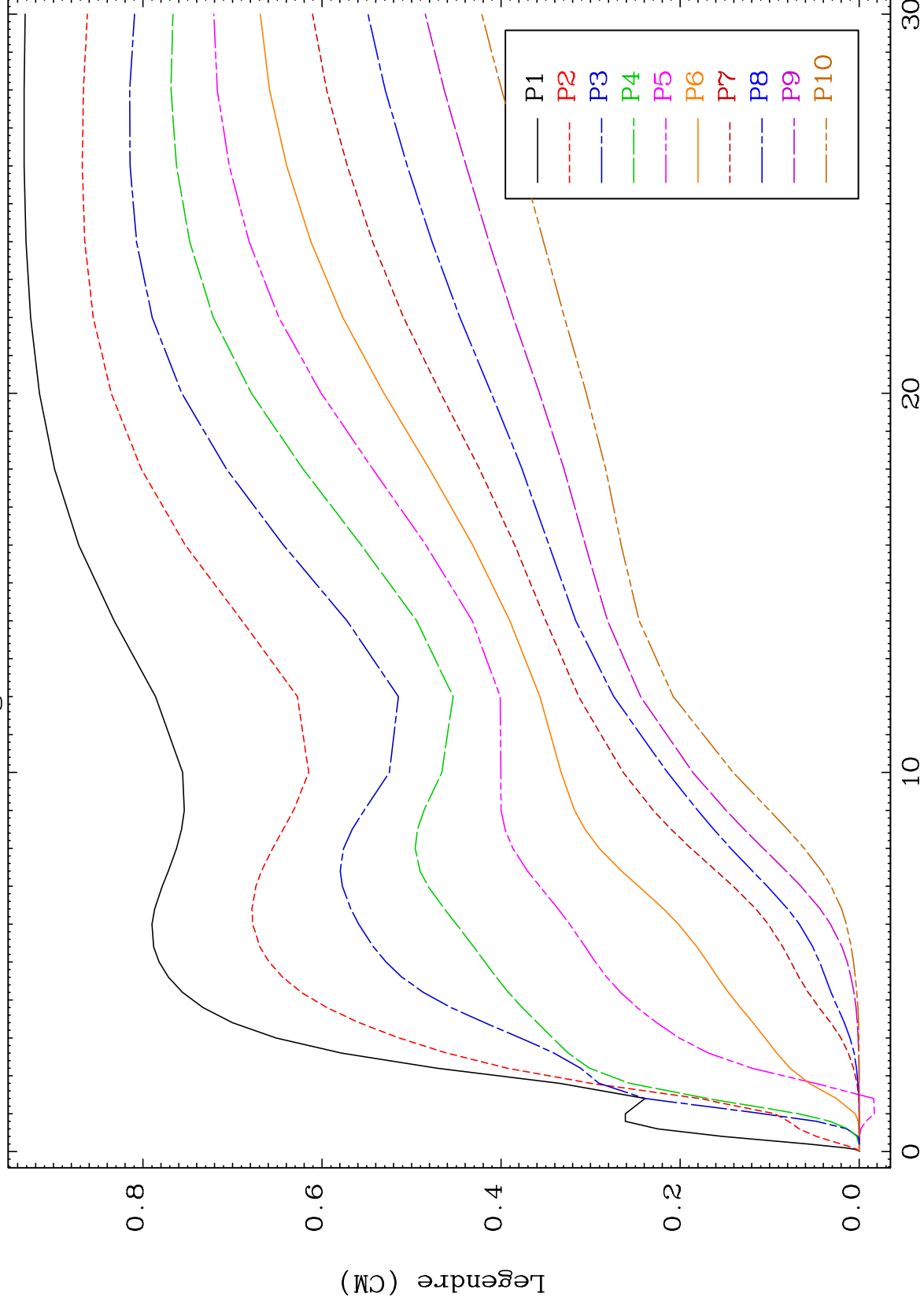
89-Ac-223



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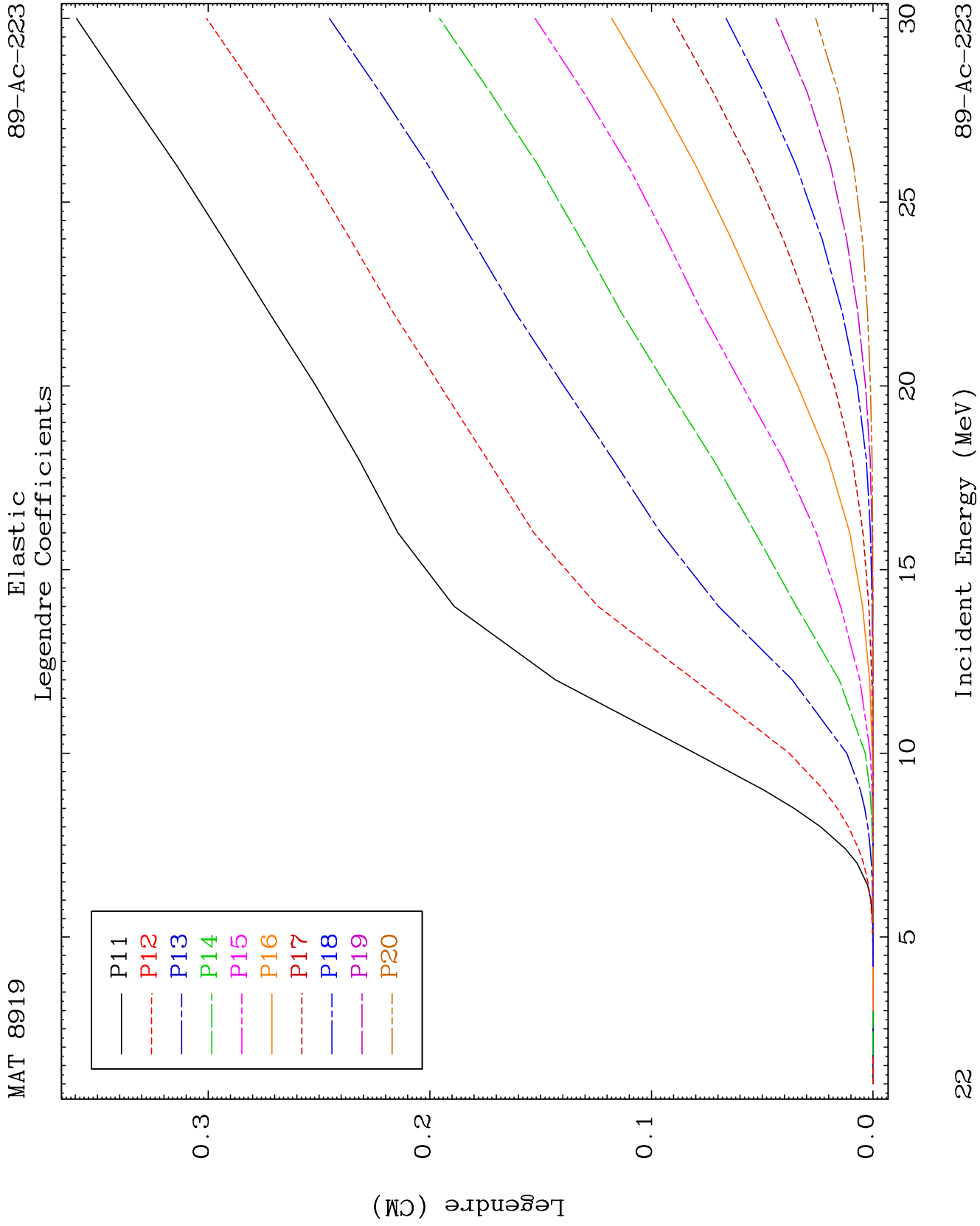
Elastic Legendre Coefficients



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Incident Energy (MeV)

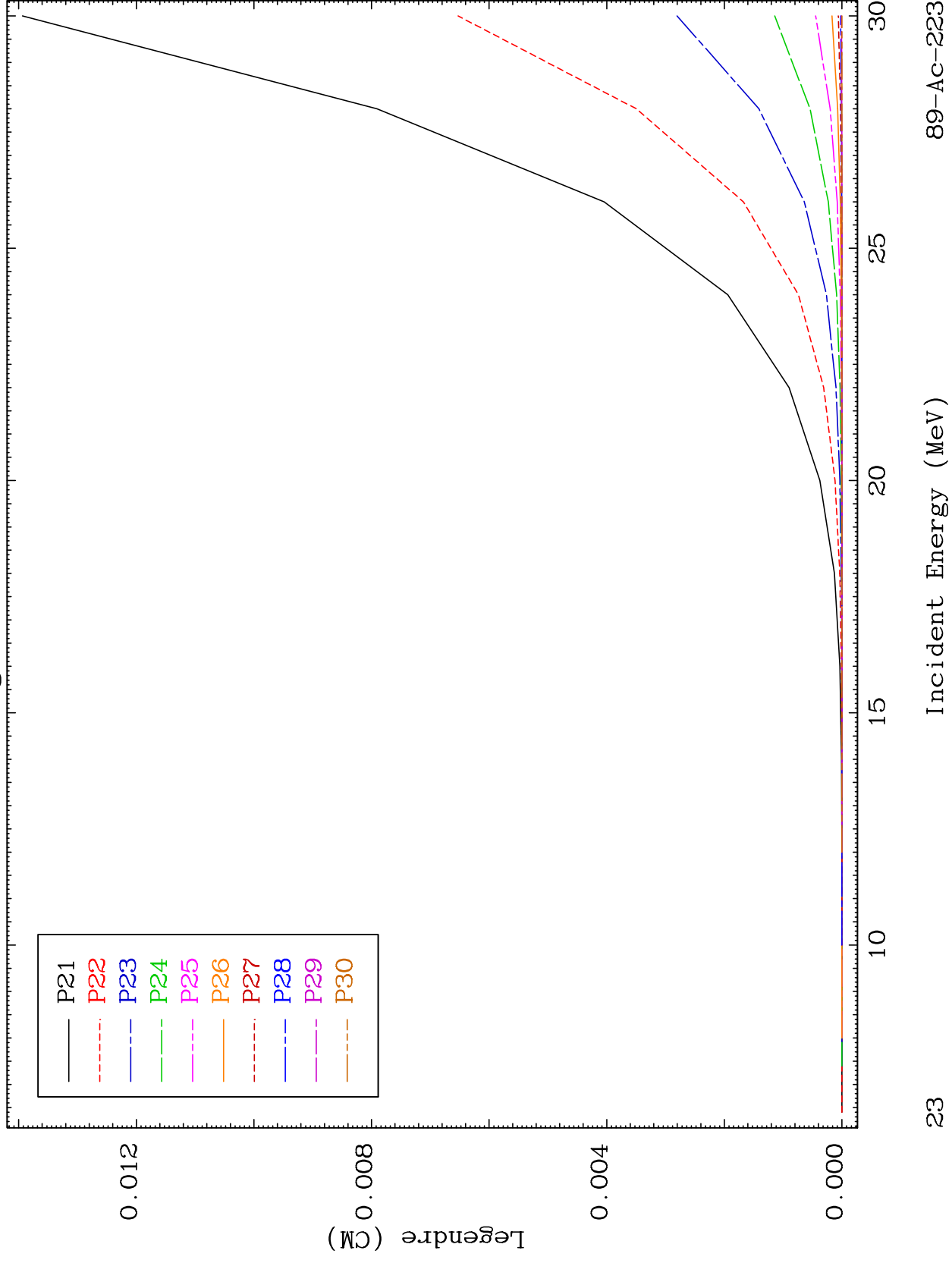
89-AC-223

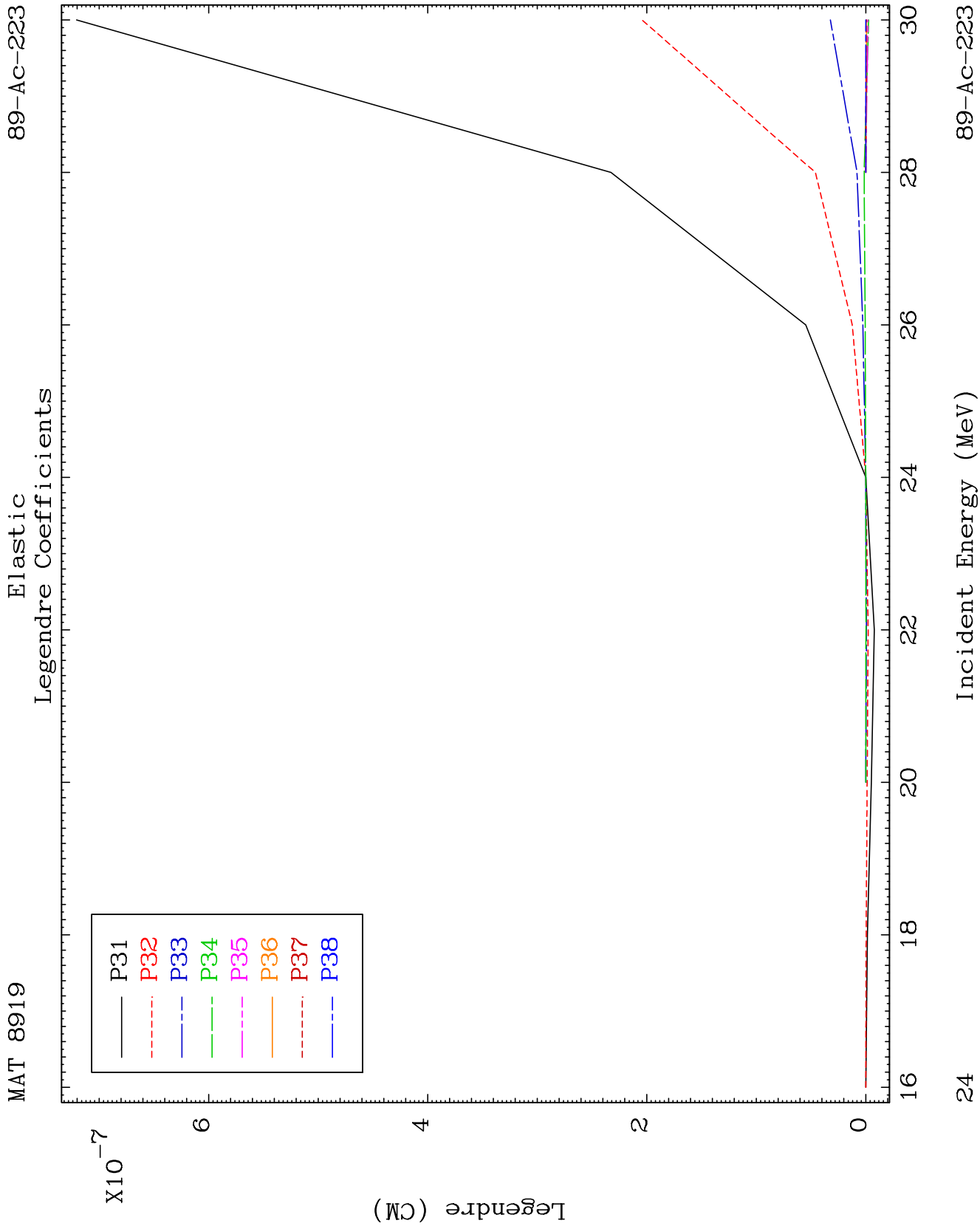


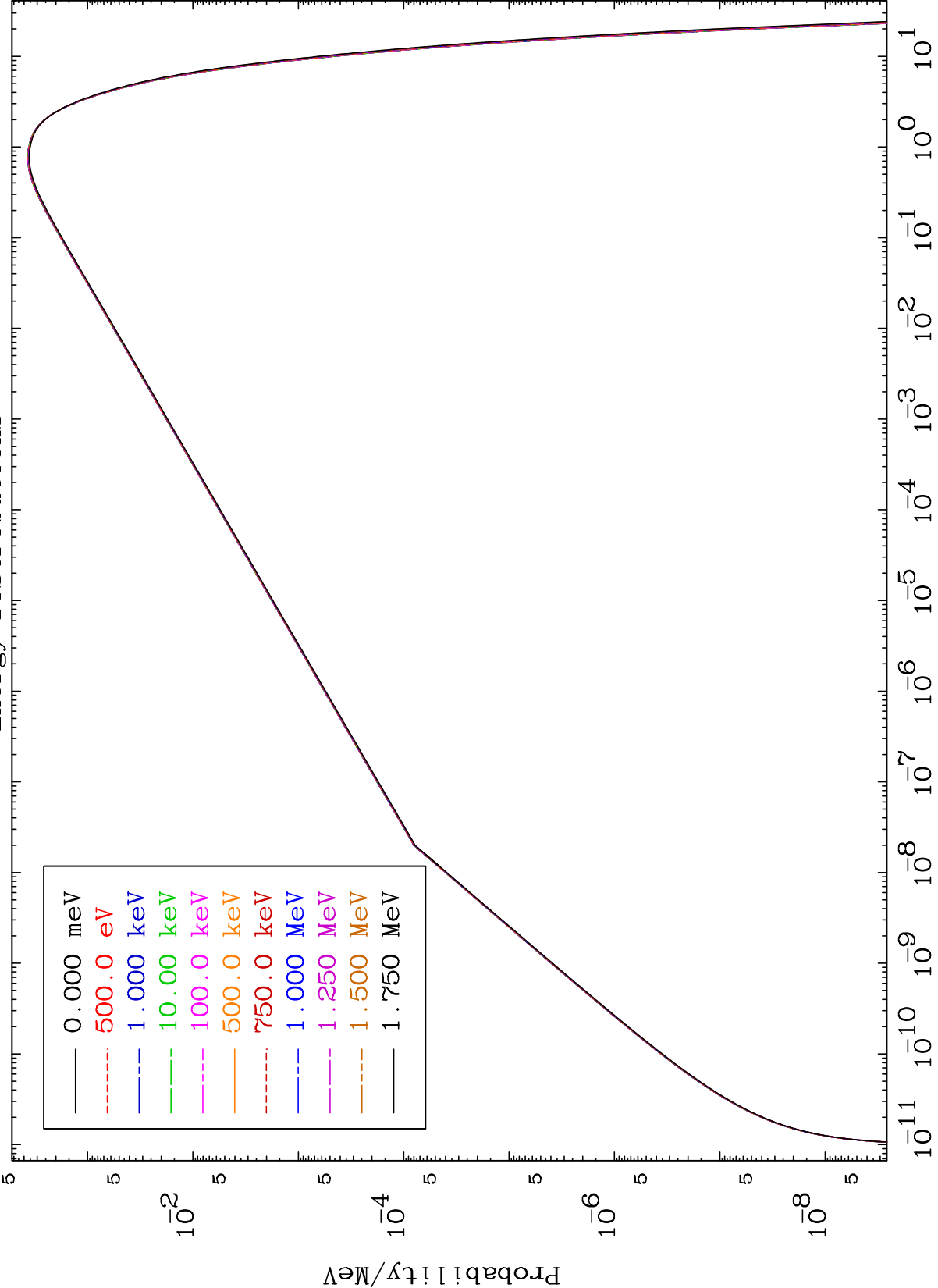
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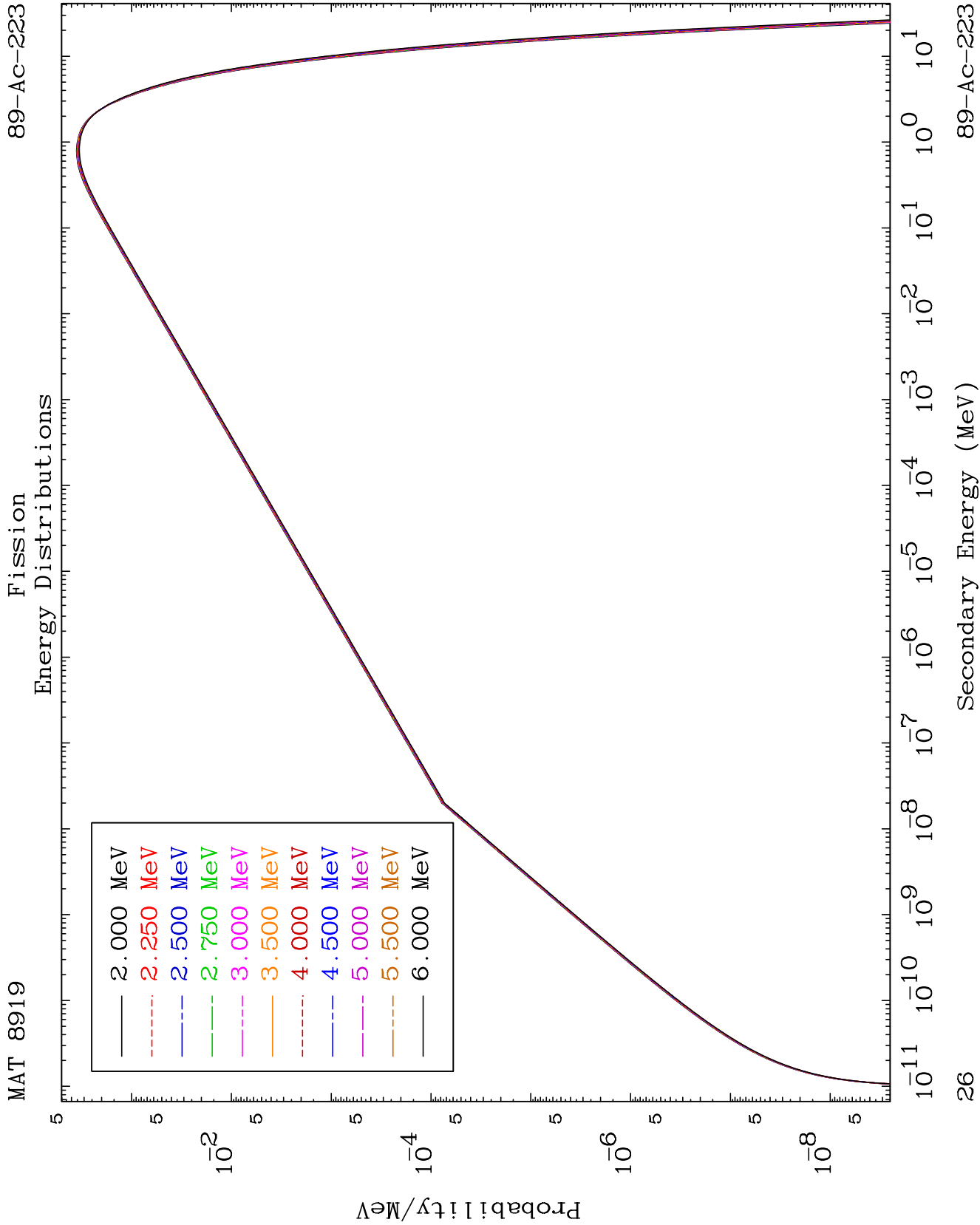
Elastic Legendre Coefficients

89-AC-223





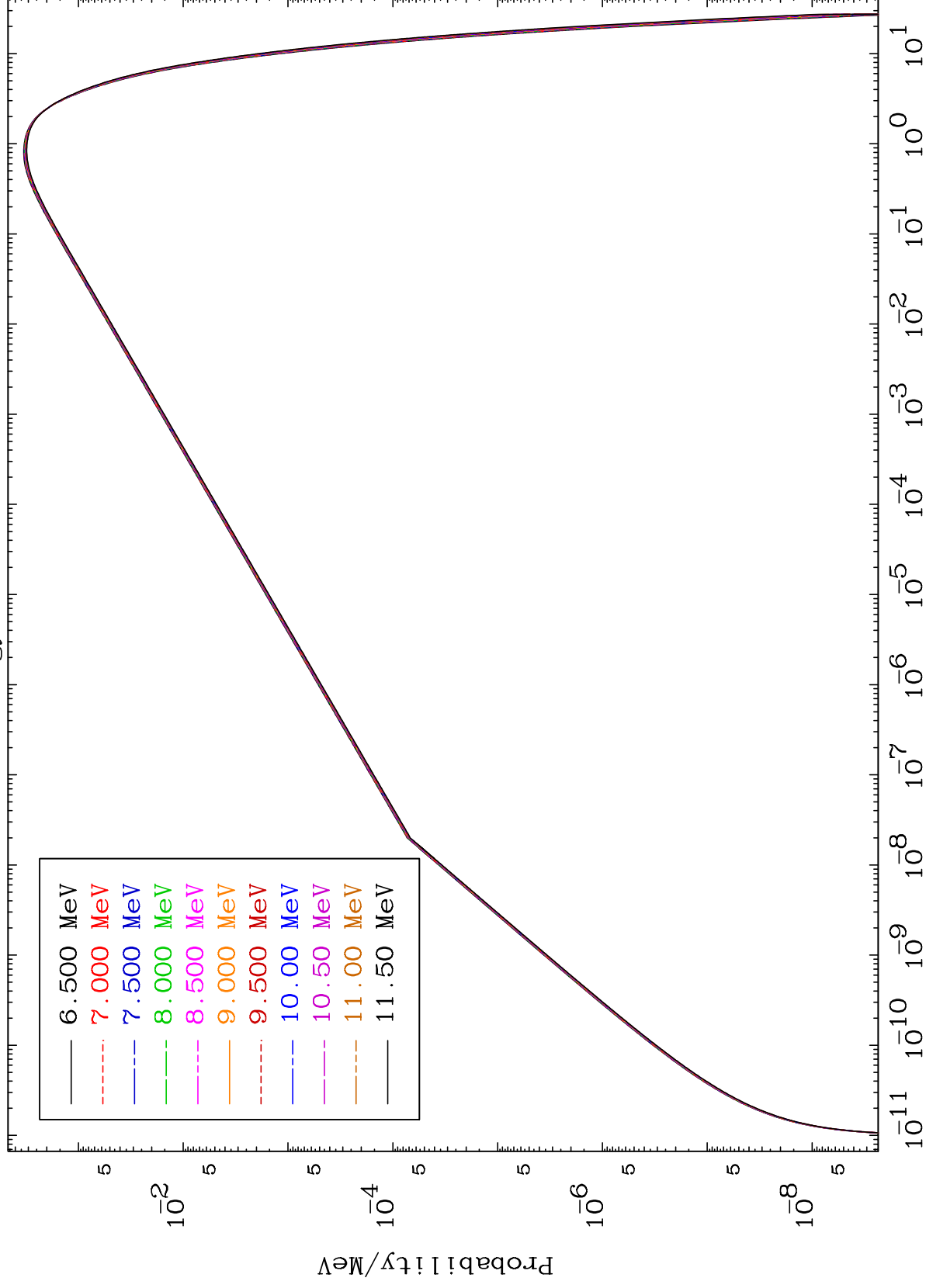


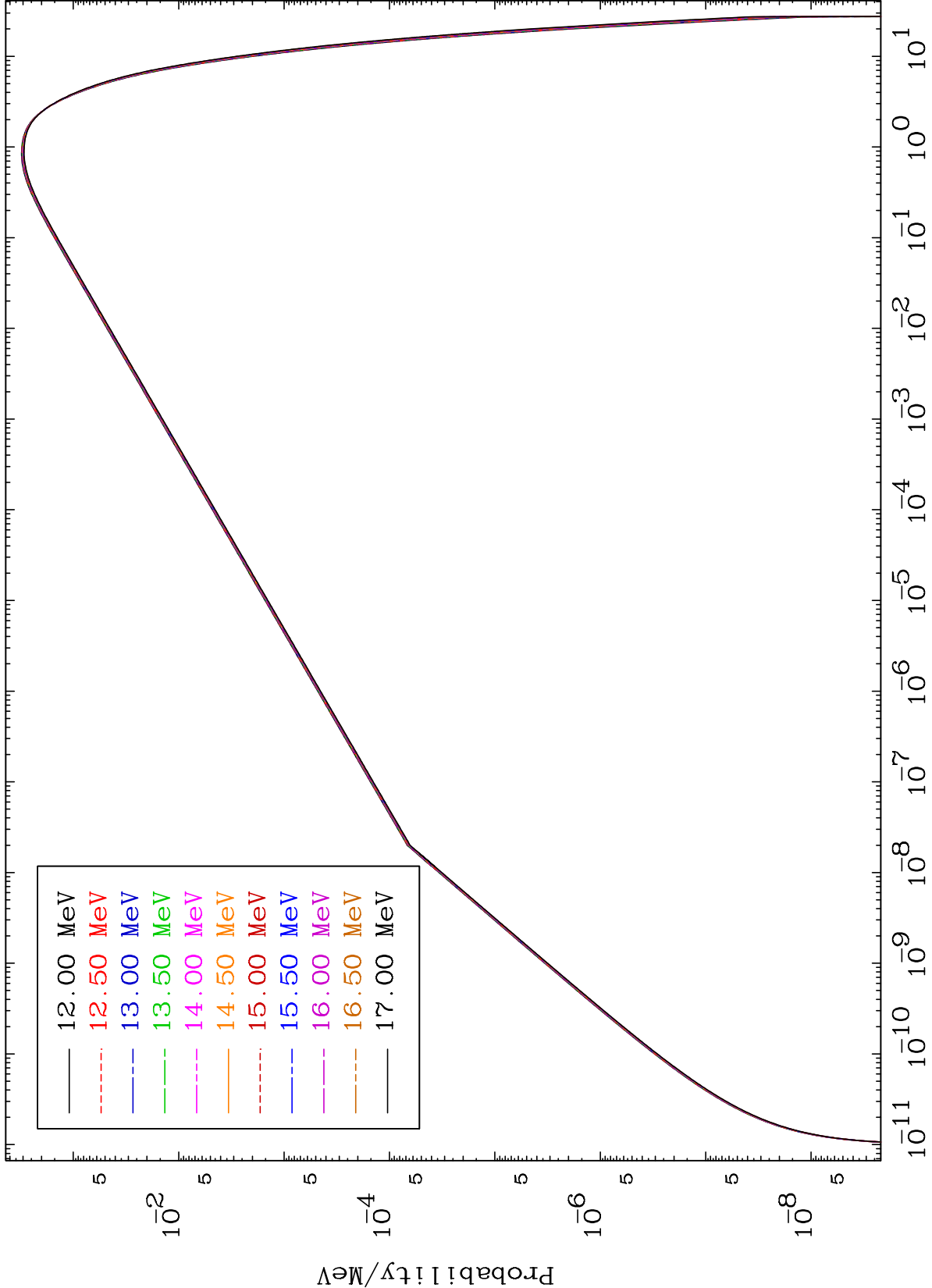


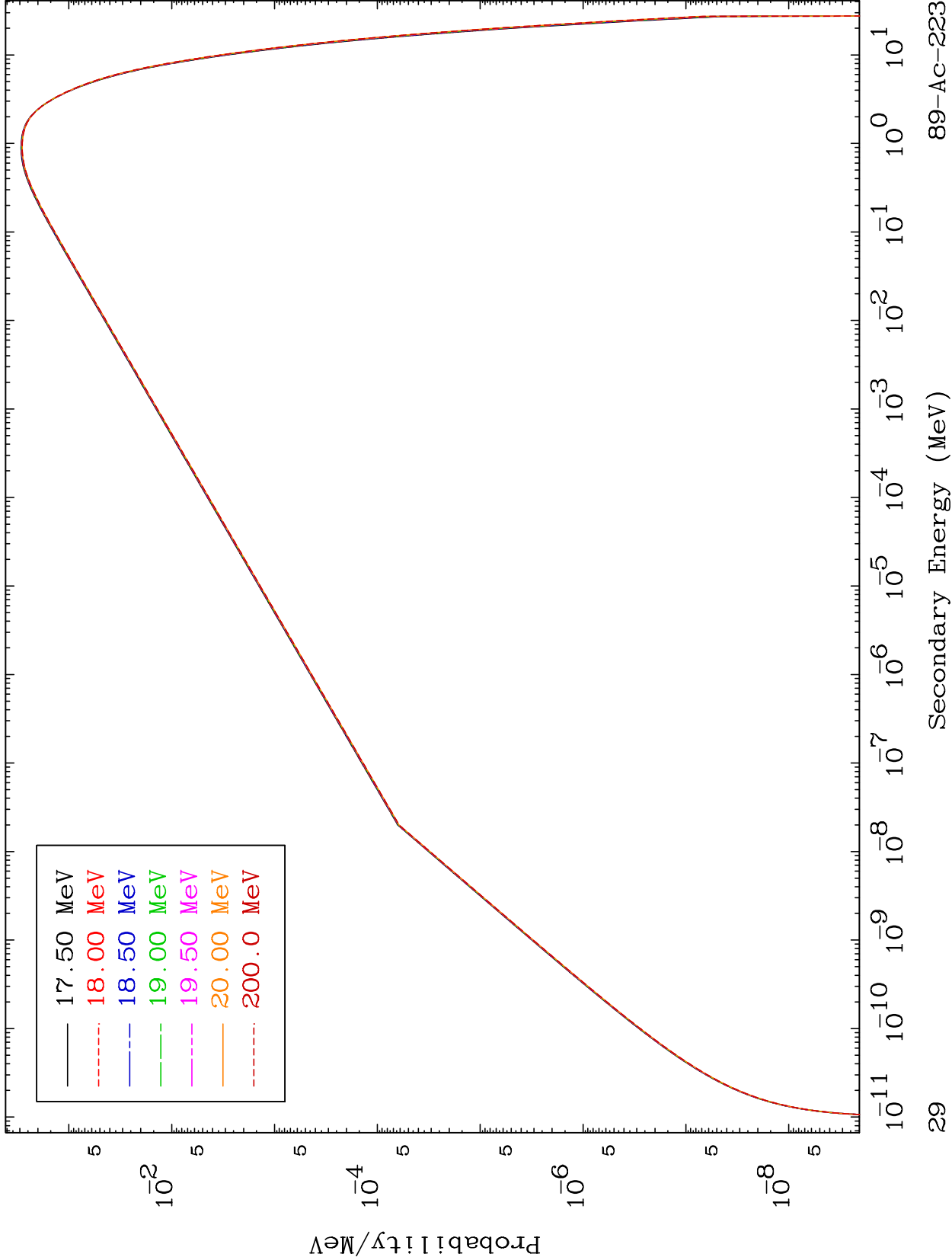
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Fission
Energy Distributions

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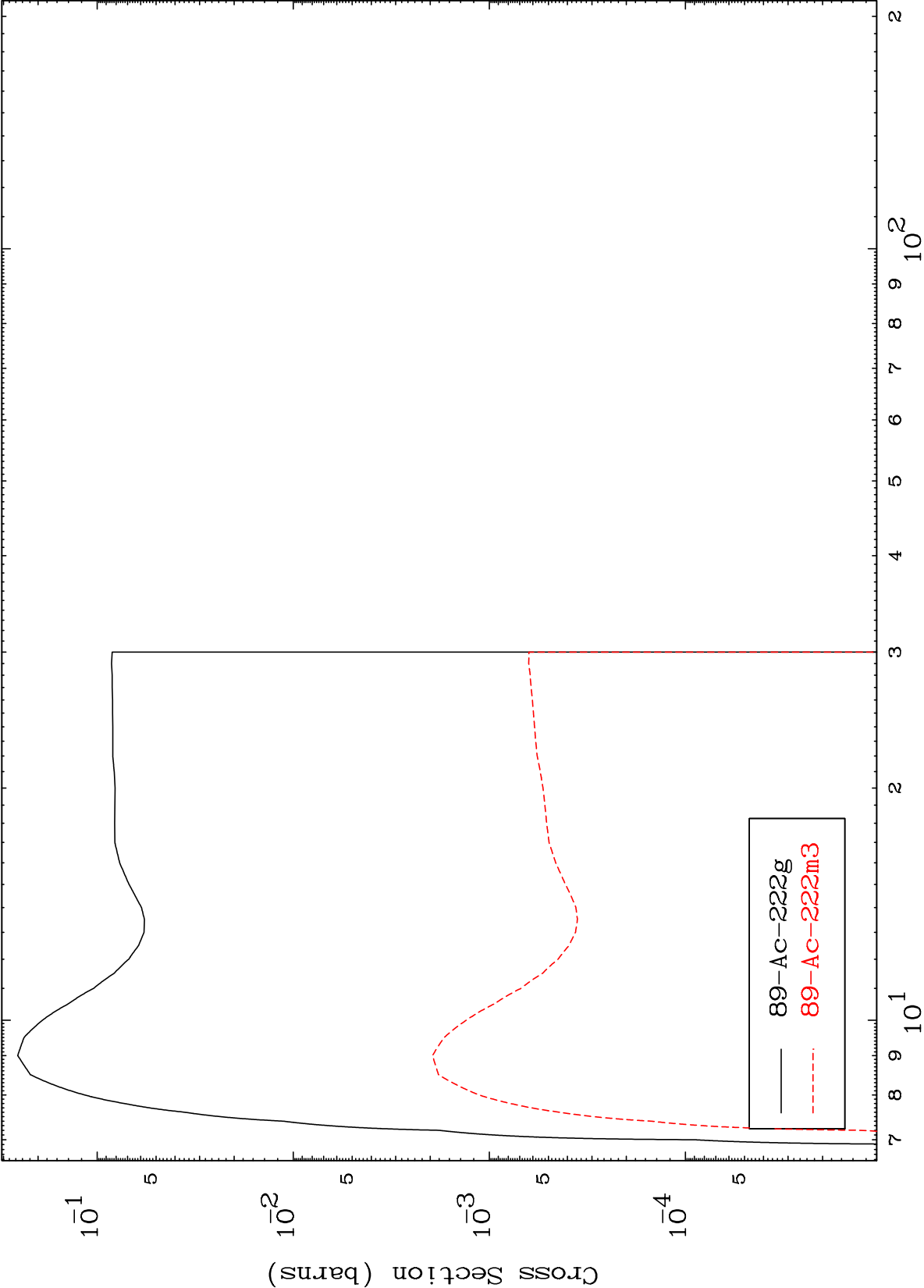




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(n,2n)
Radionuclide Production Cross Section



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Incident Energy (MeV)

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