

Program EVALPLOT
(Version 2021-1)

by

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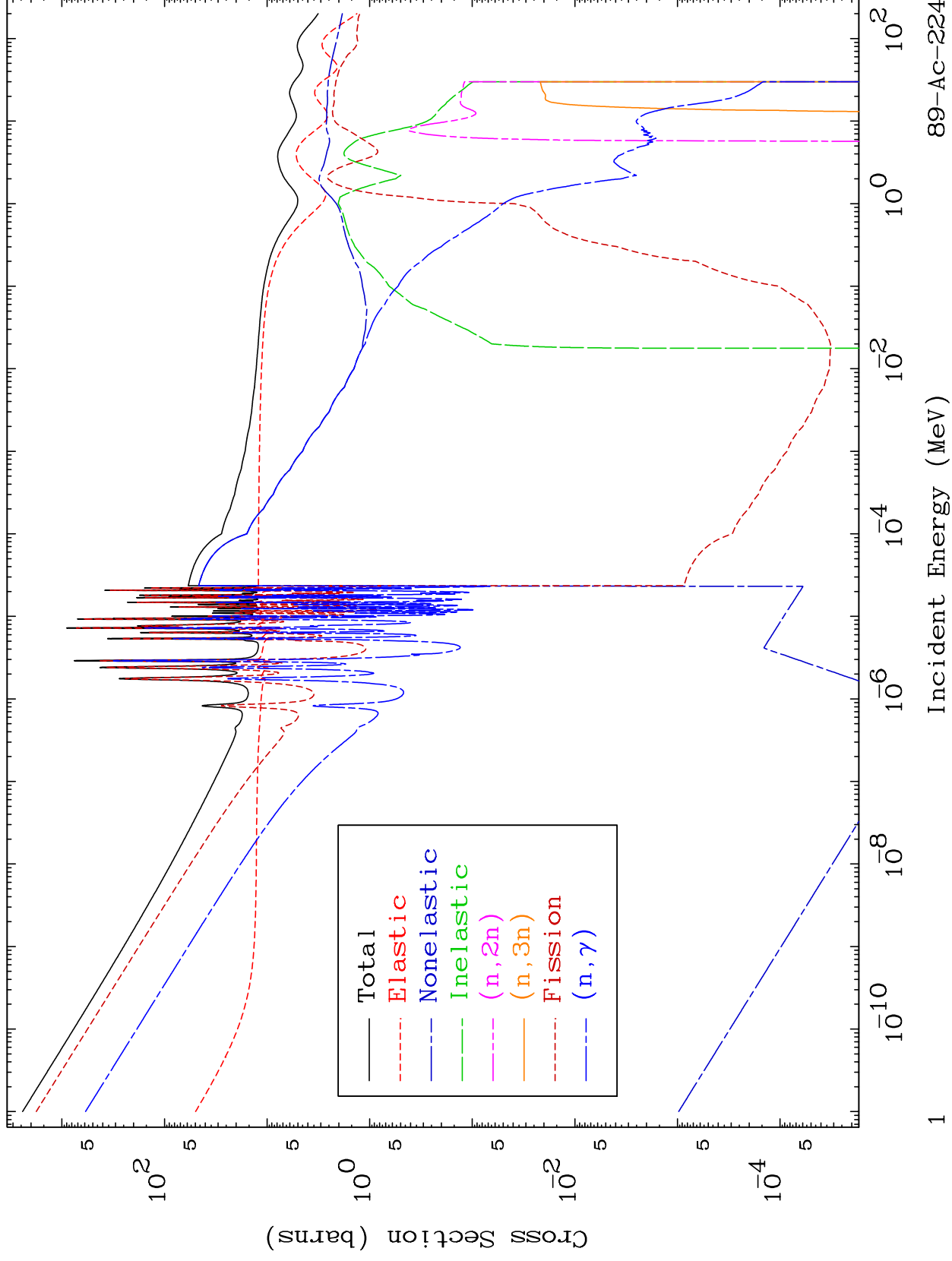
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

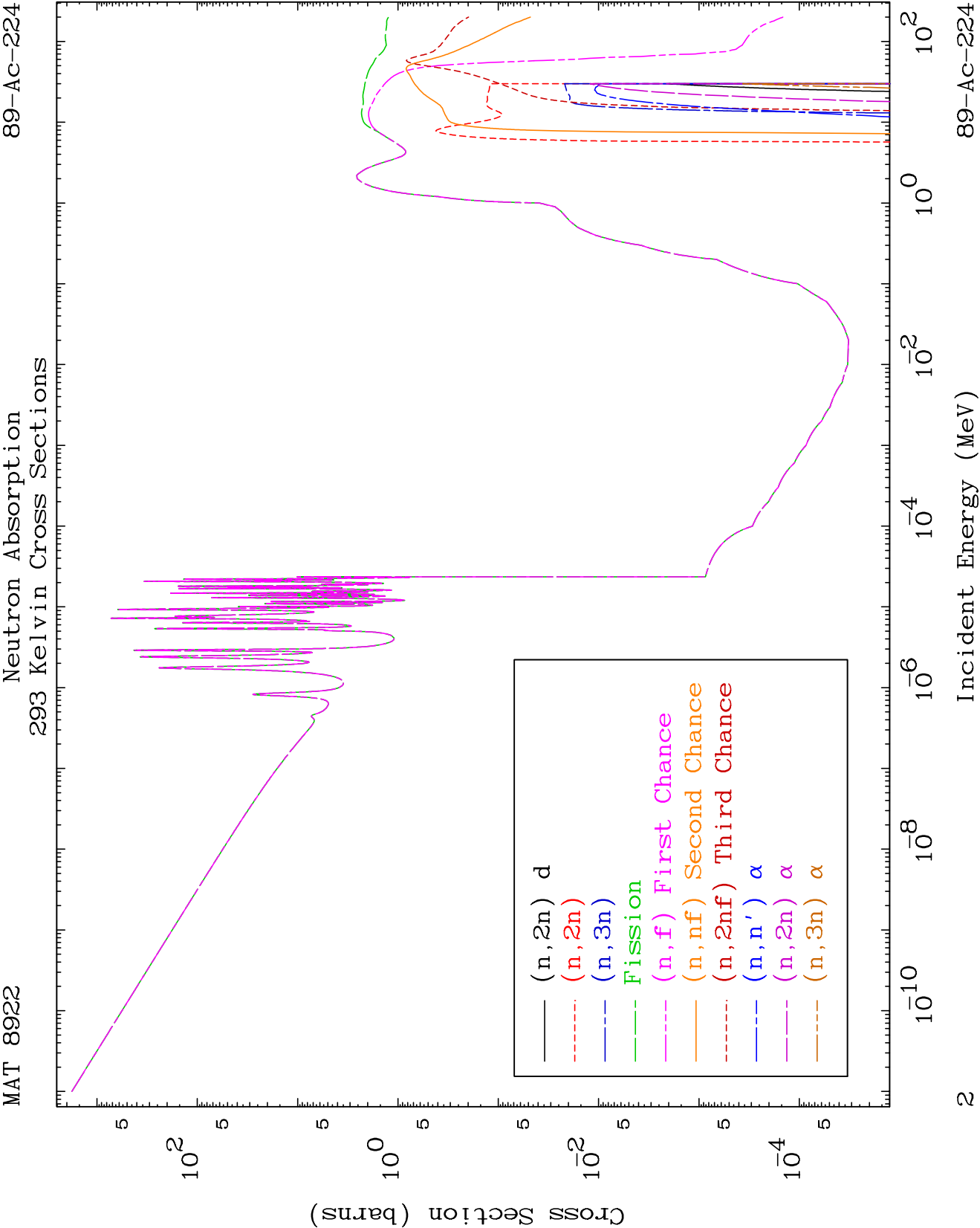
Press Mouse Button to Start

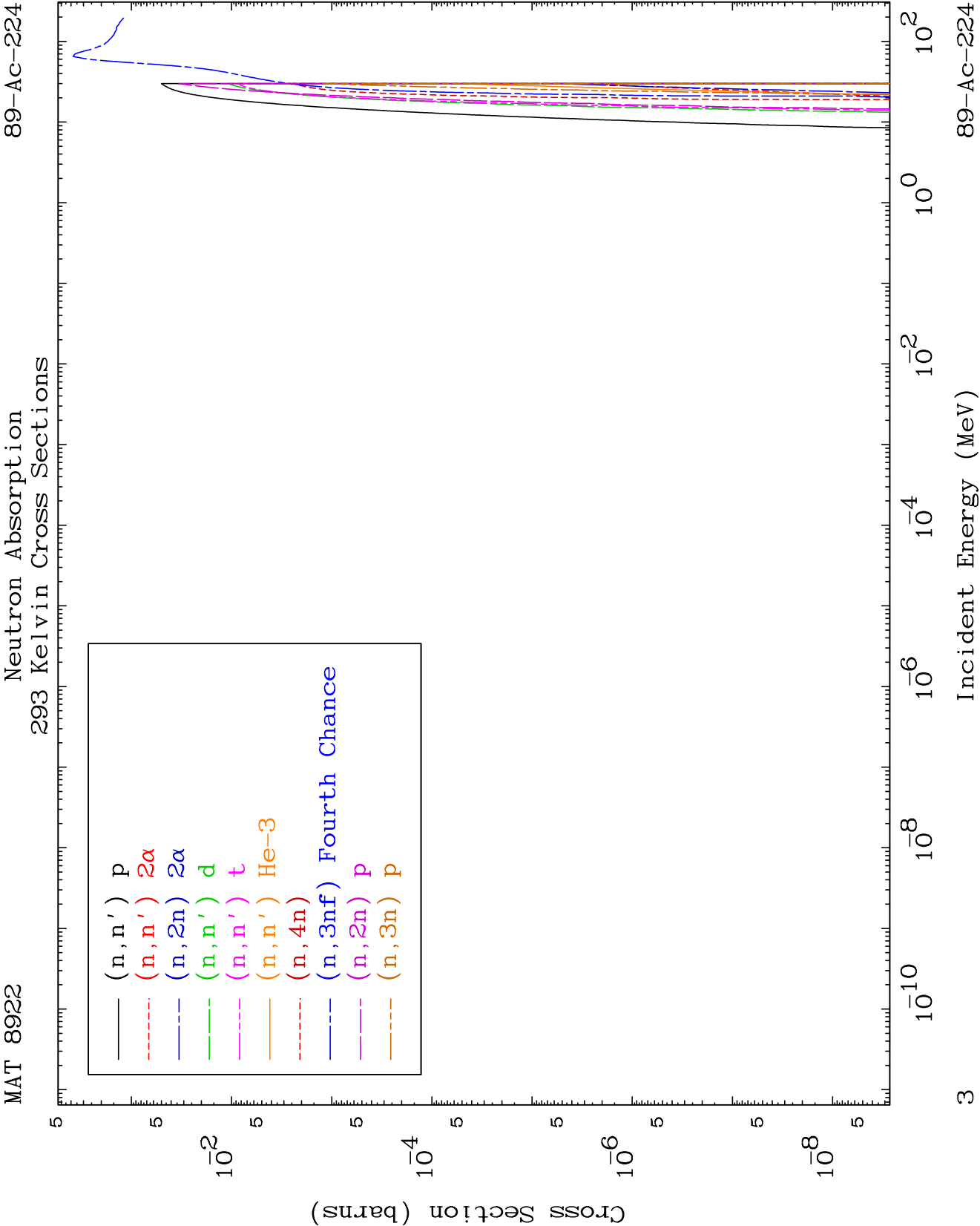
MAT 8922

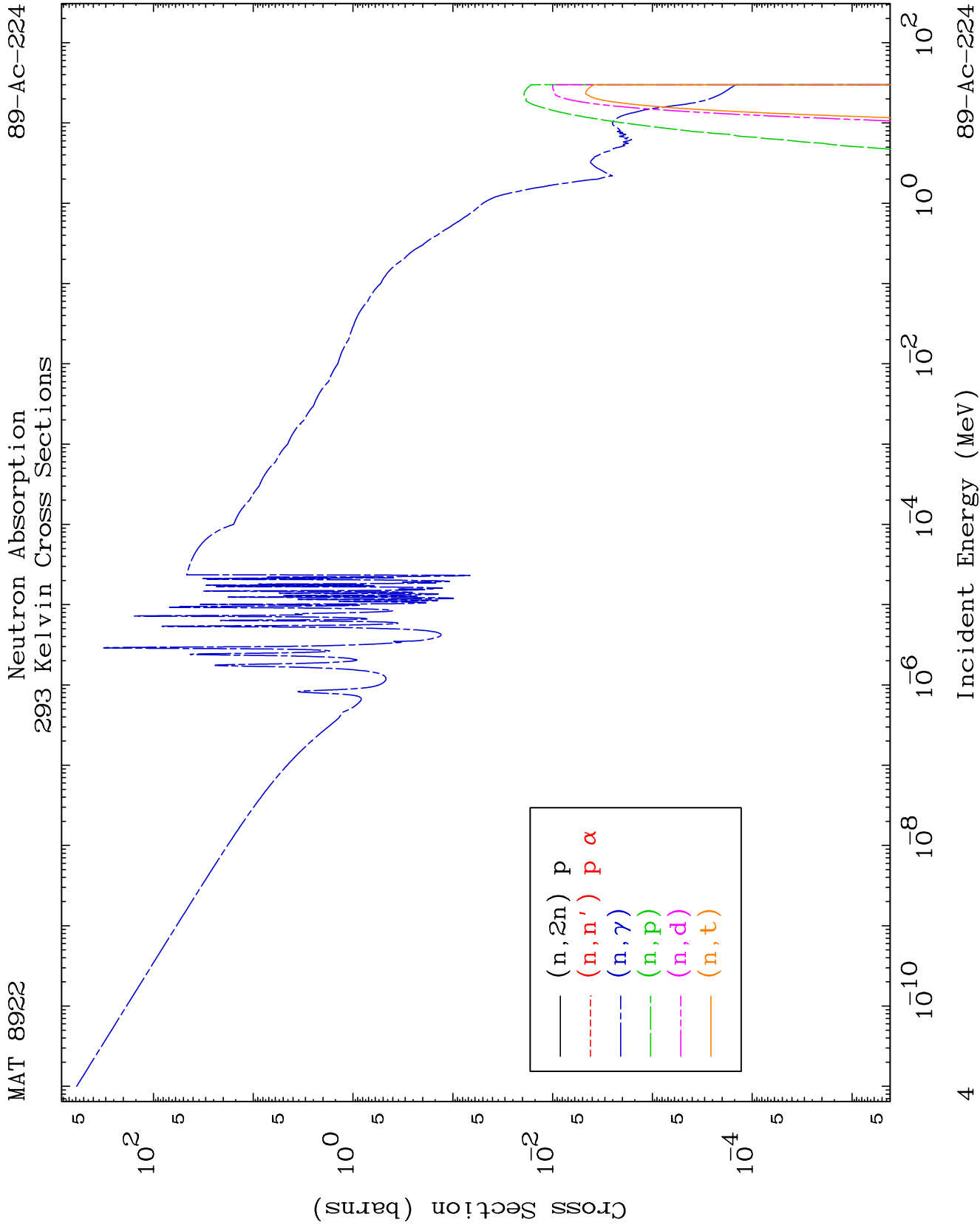
Neutron Major
293 Kelvin Cross Sections

89-Ac-224





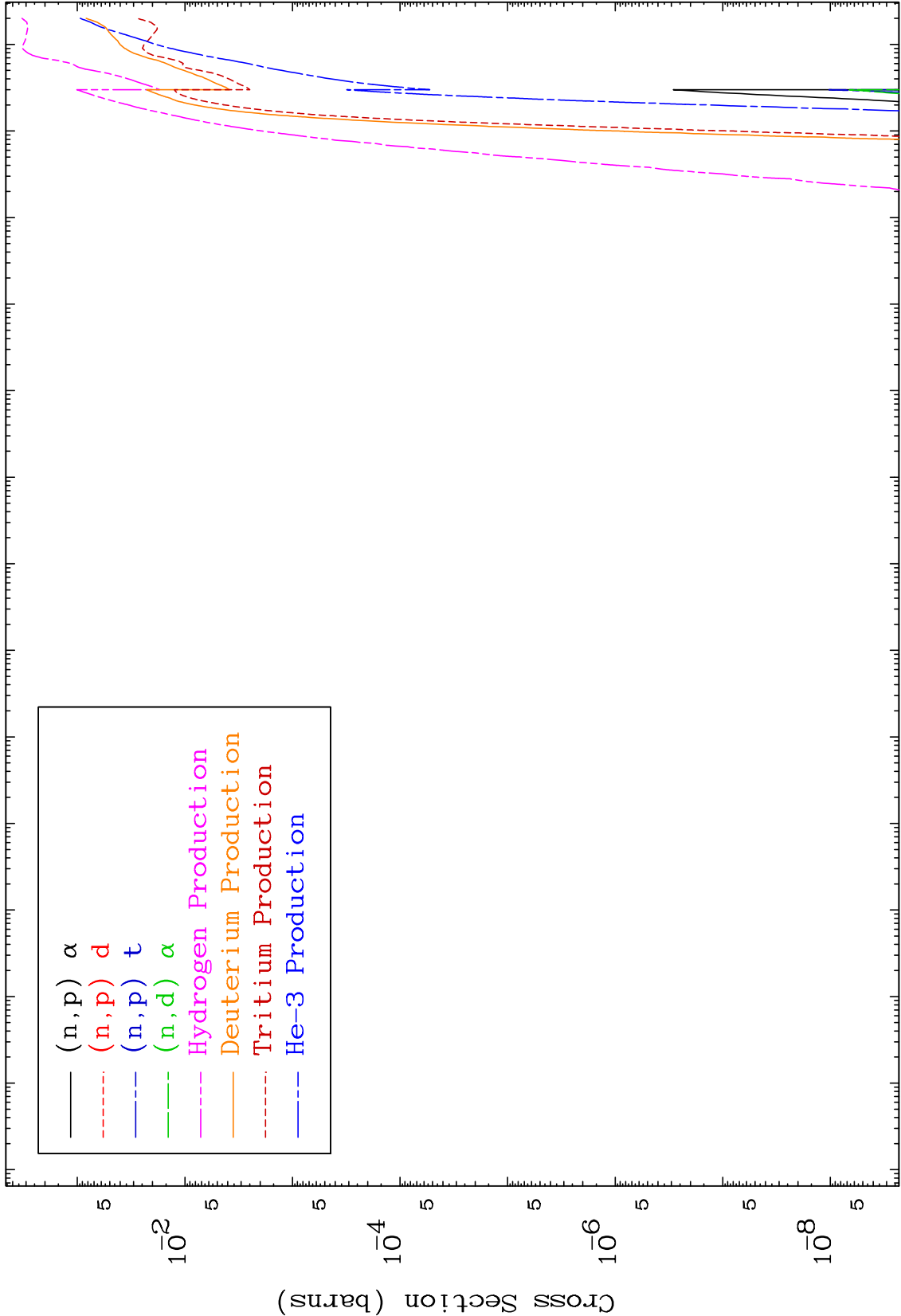




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Neutron Absorption
293 Kelvin Cross Sections

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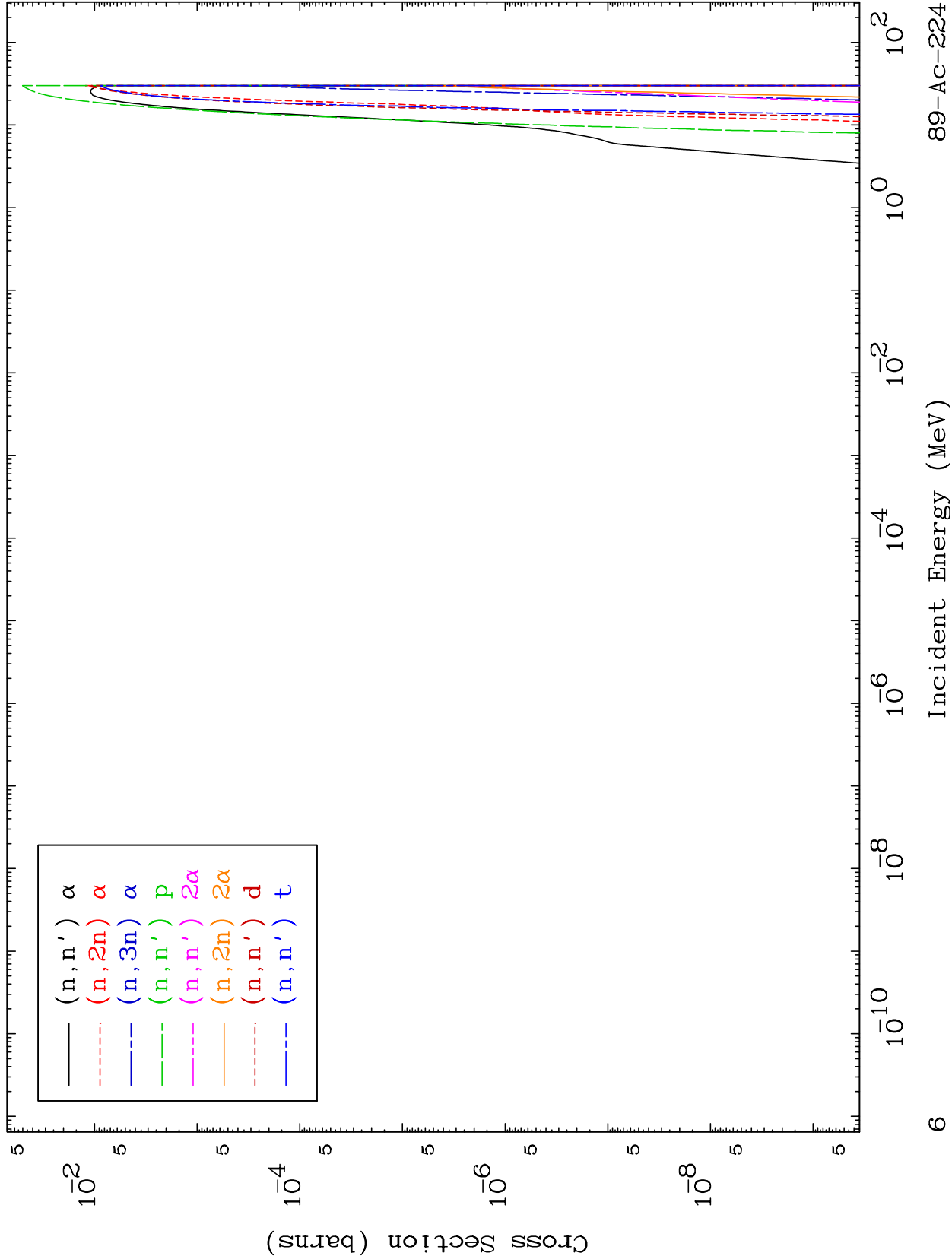
Incident Energy (MeV)

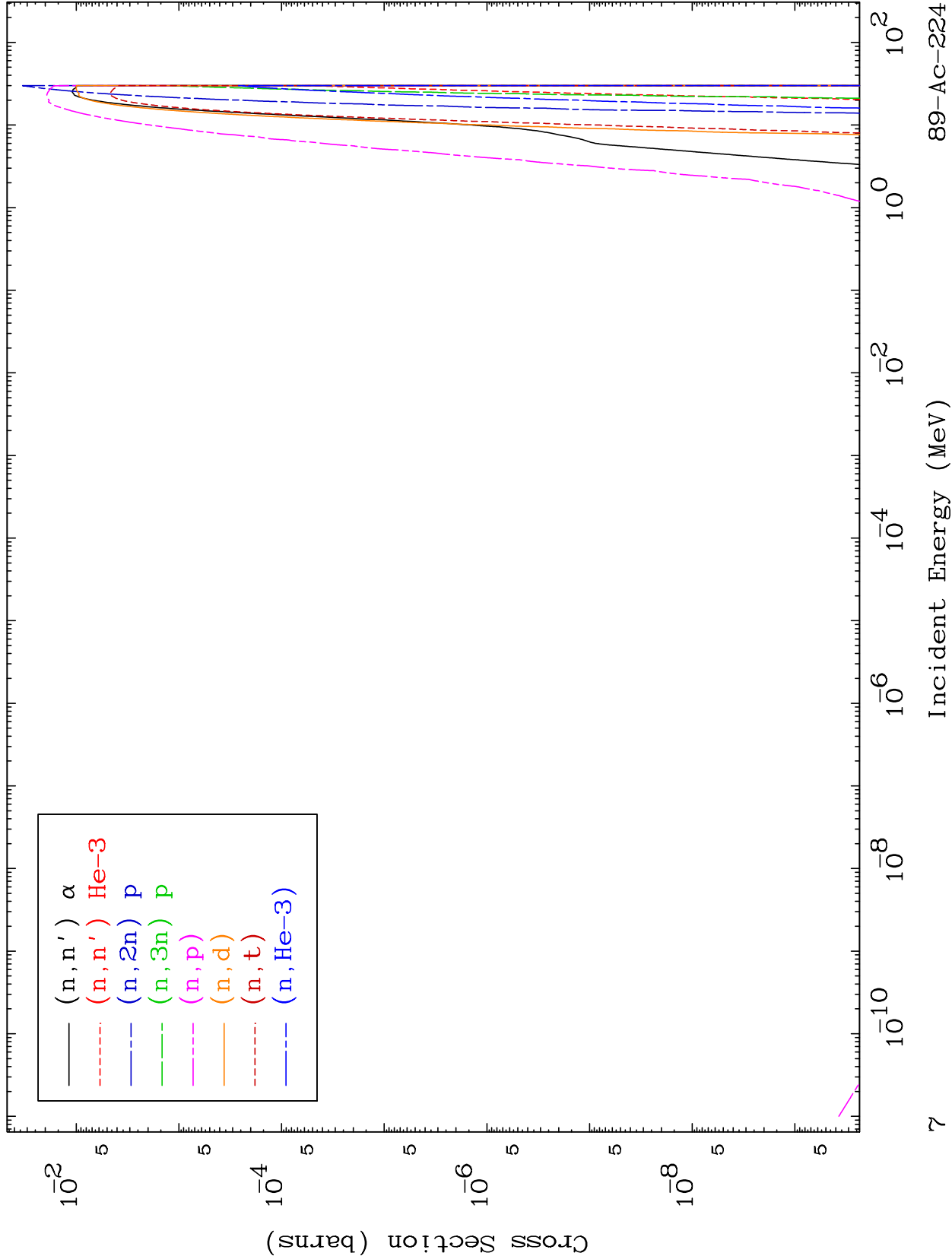
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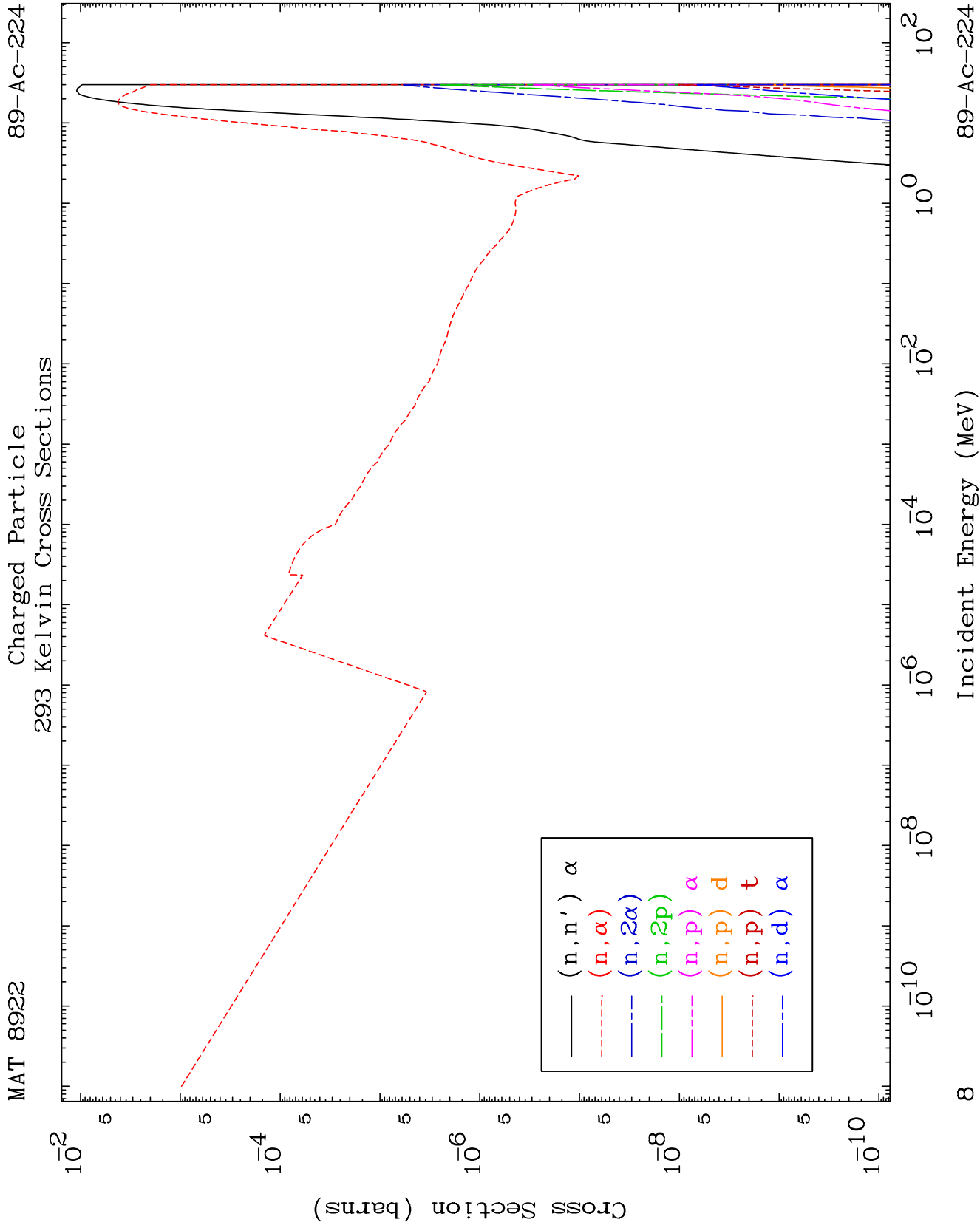
MAT 8922

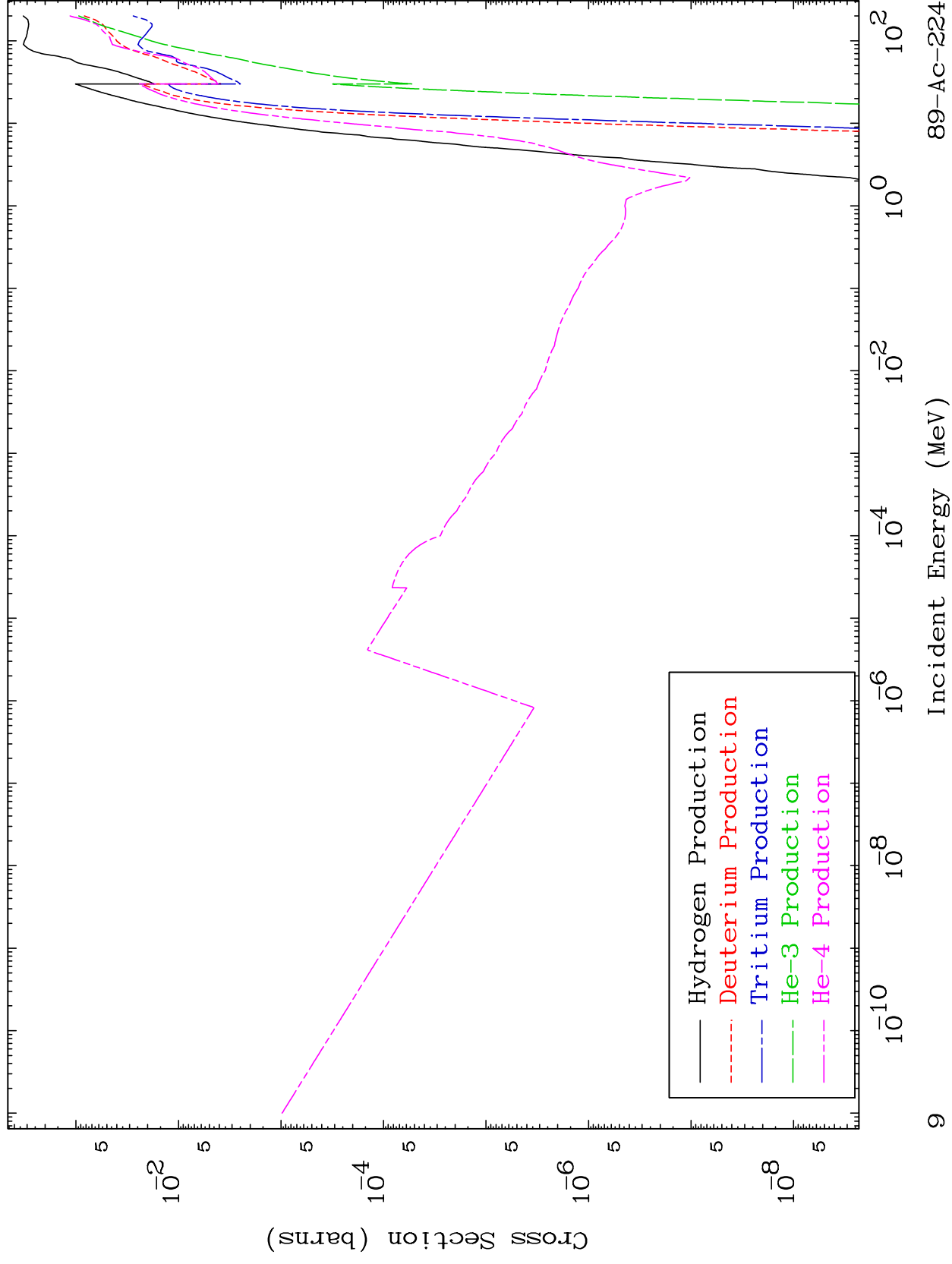
Charged Particle
293 Kelvin Cross Sections

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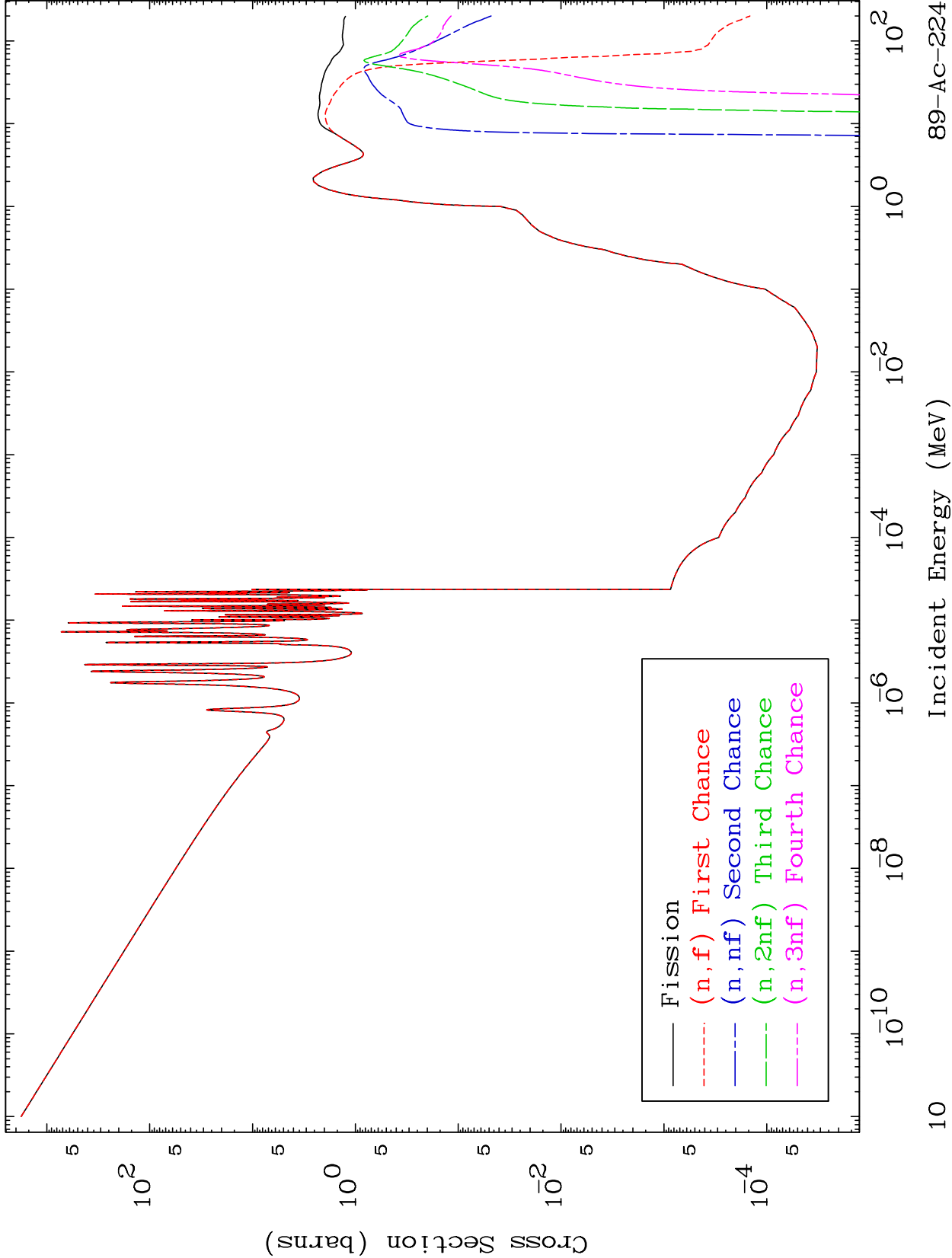


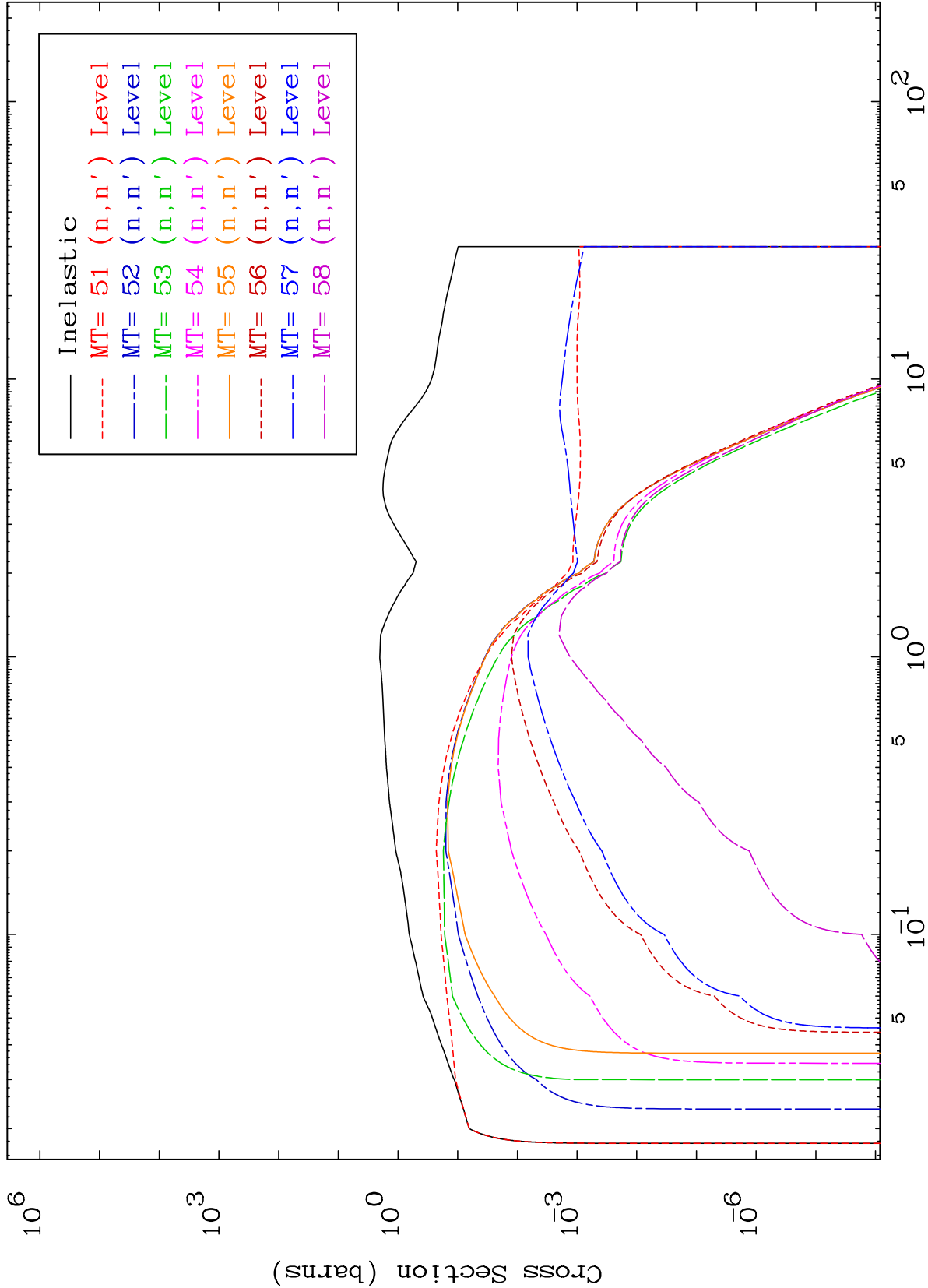


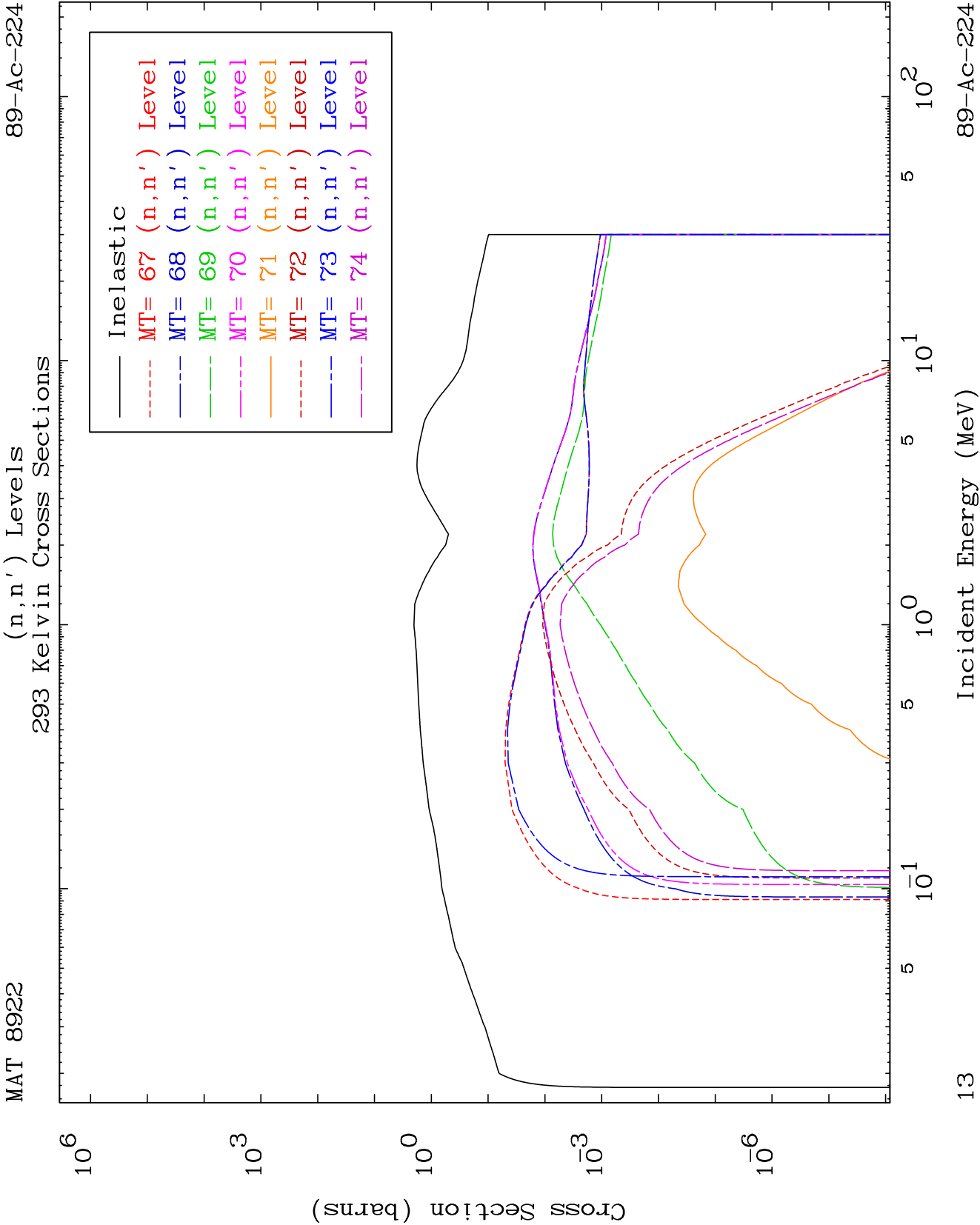
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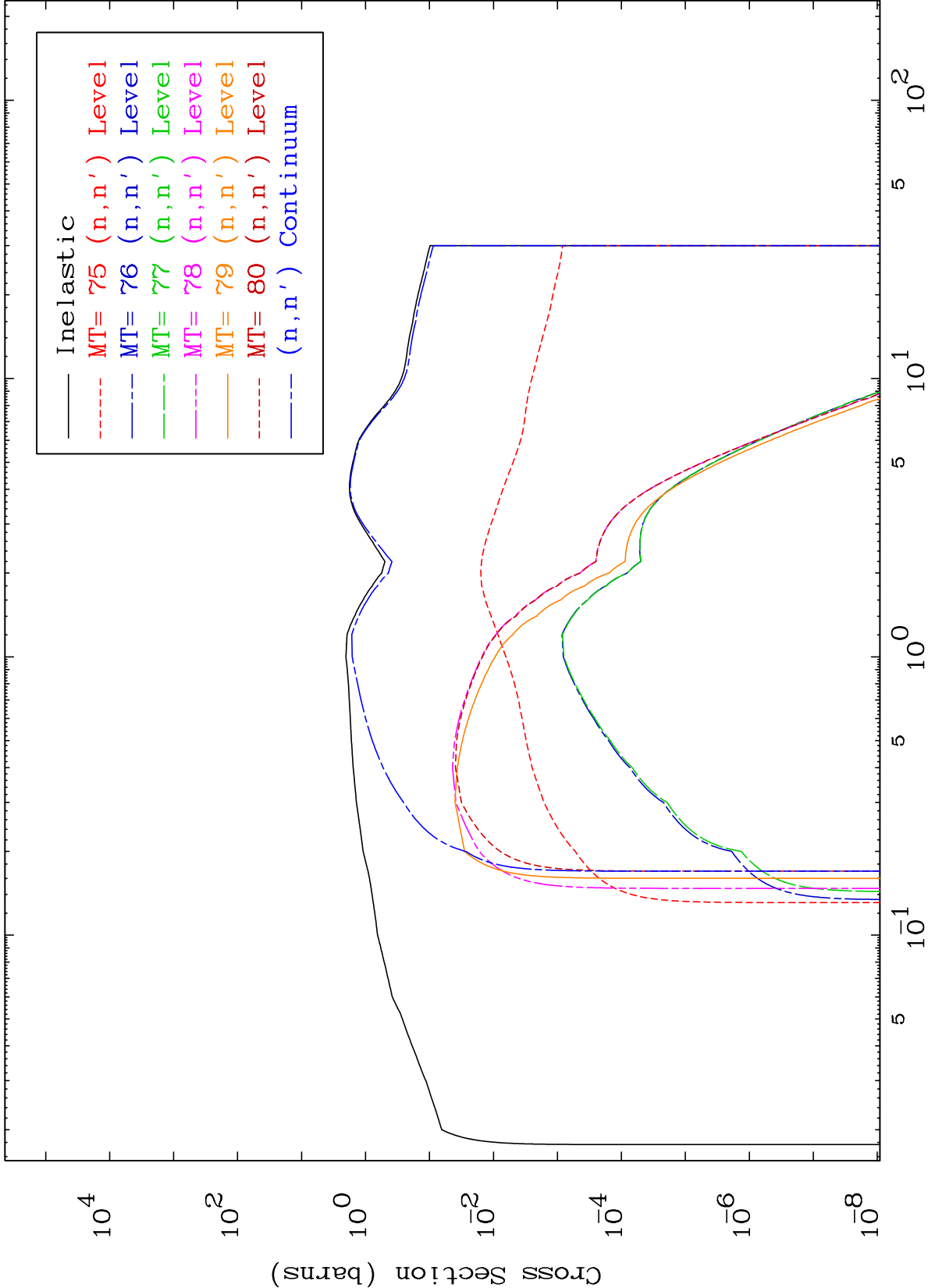
Fission
293 Kelvin Cross Sections

89-Ac-224





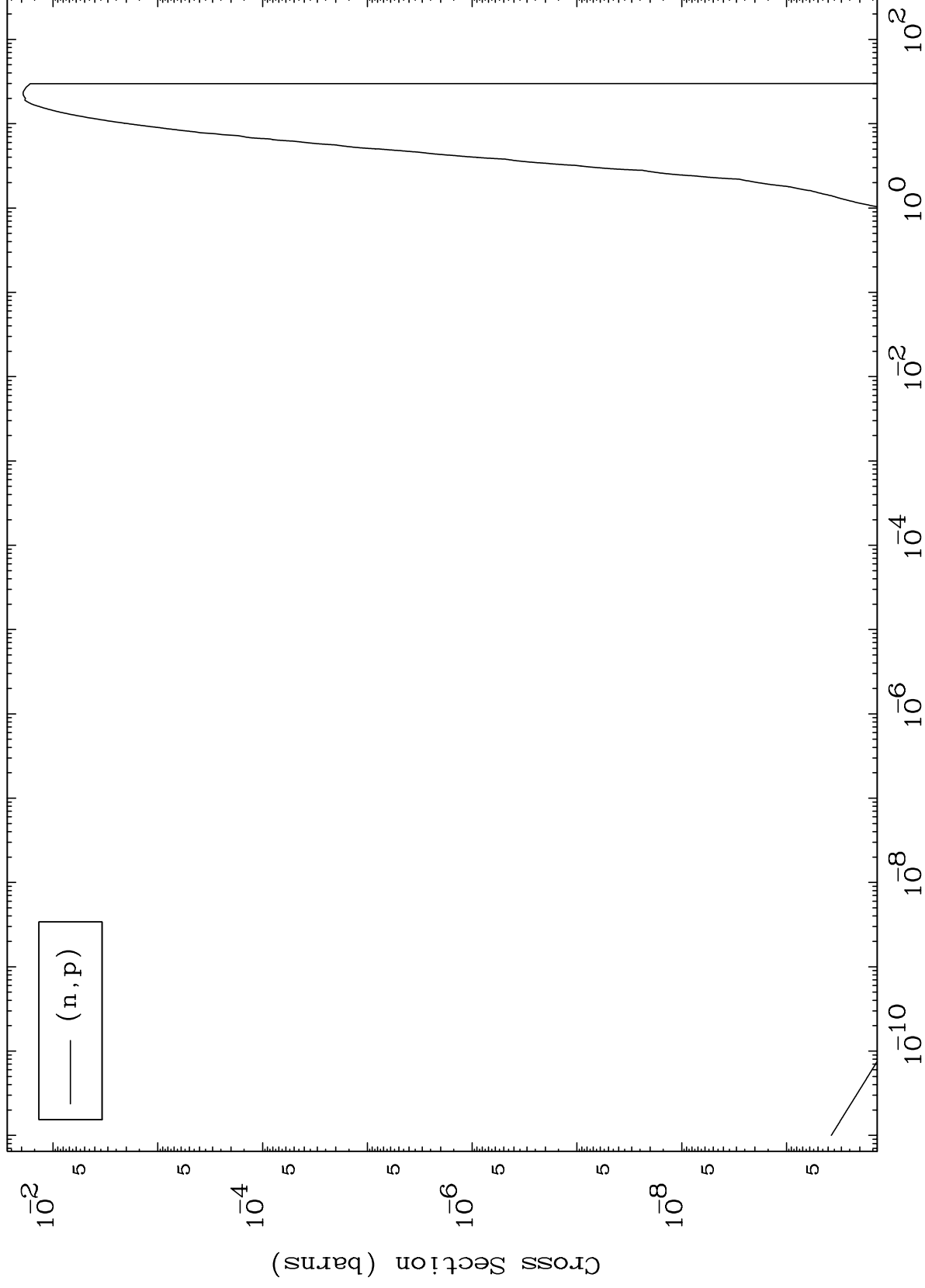




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(n,p) Levels
293 Kelvin Cross Sections

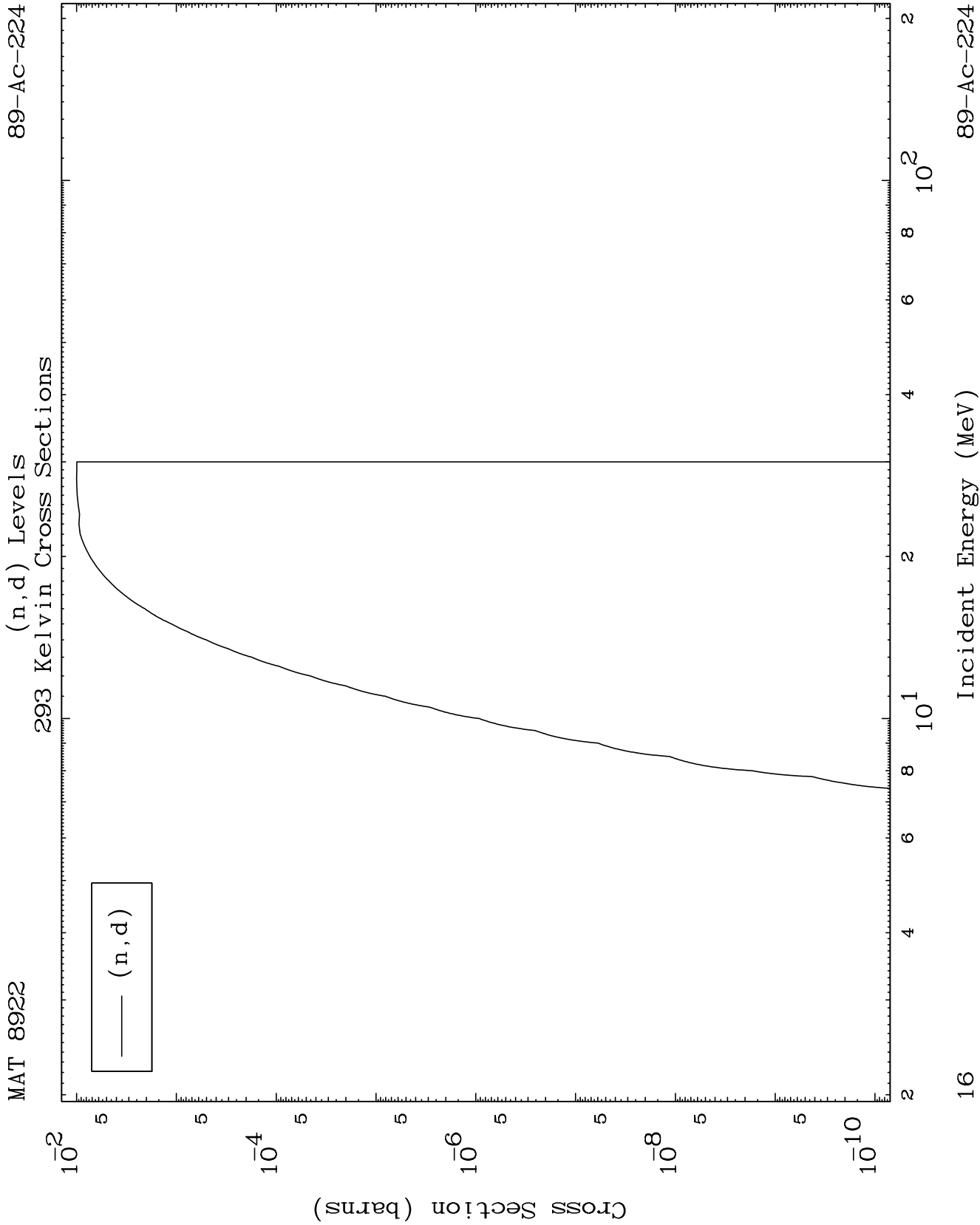
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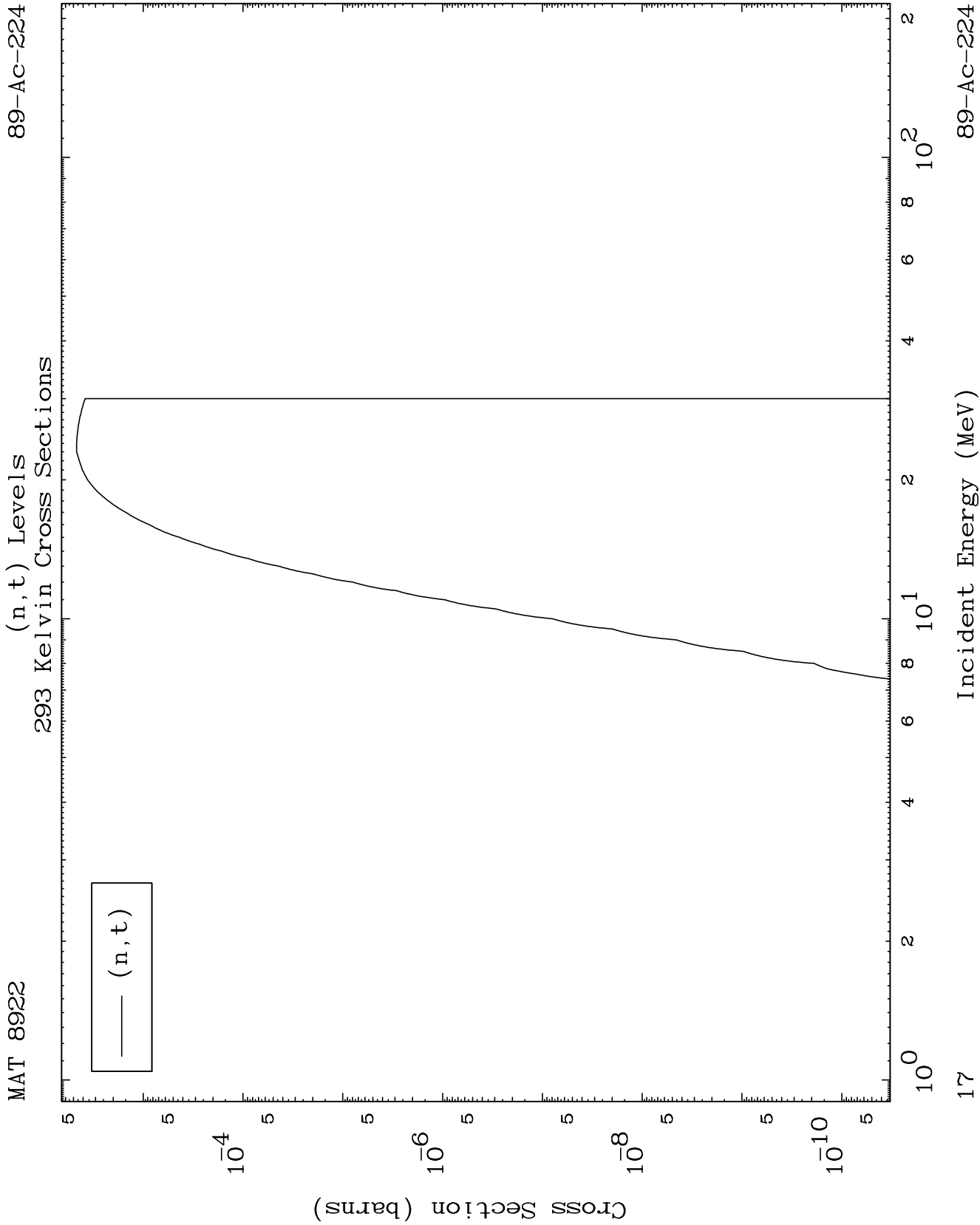


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Incident Energy (MeV)

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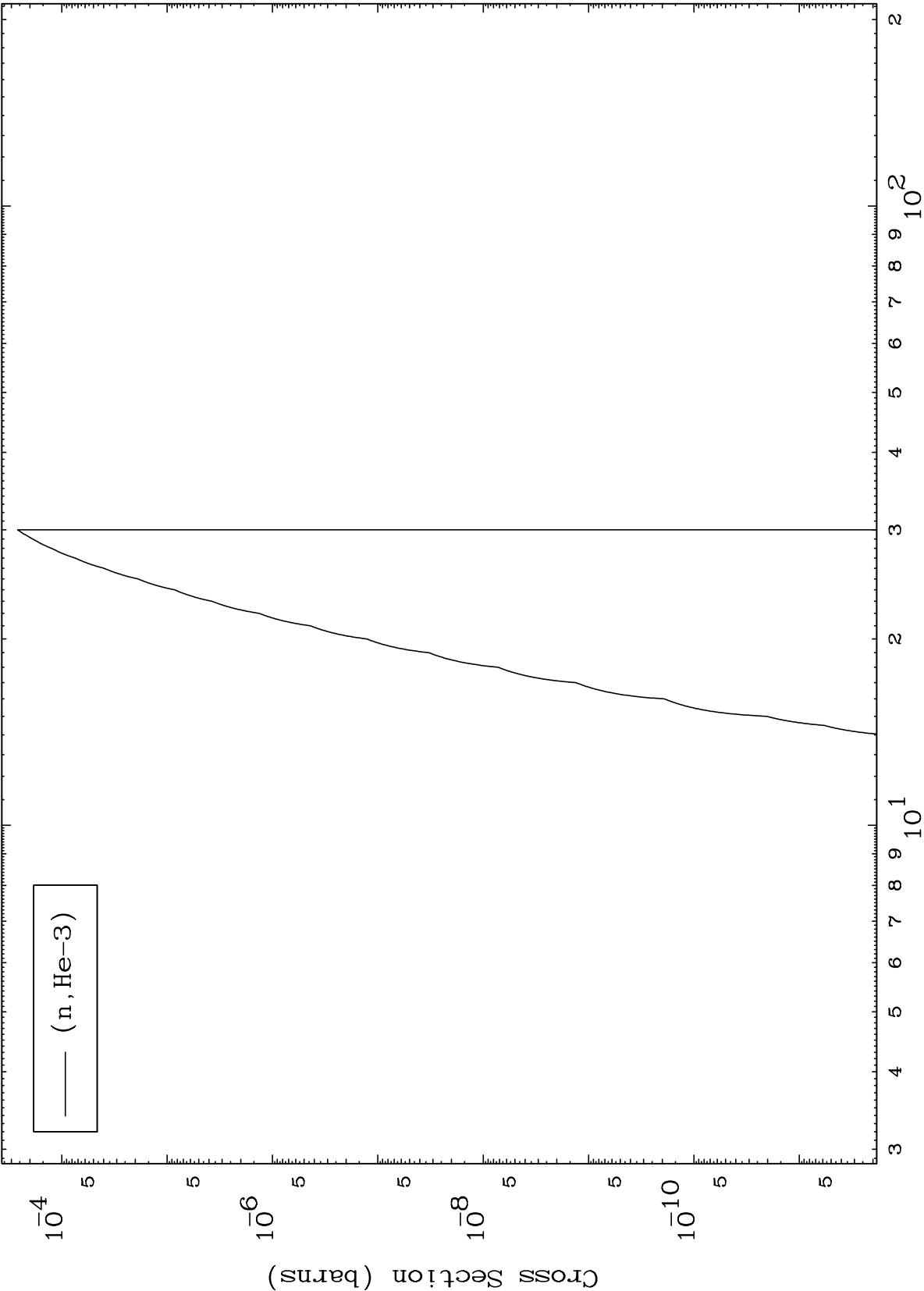




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(n,He3) Levels
293 Kelvin Cross Sections

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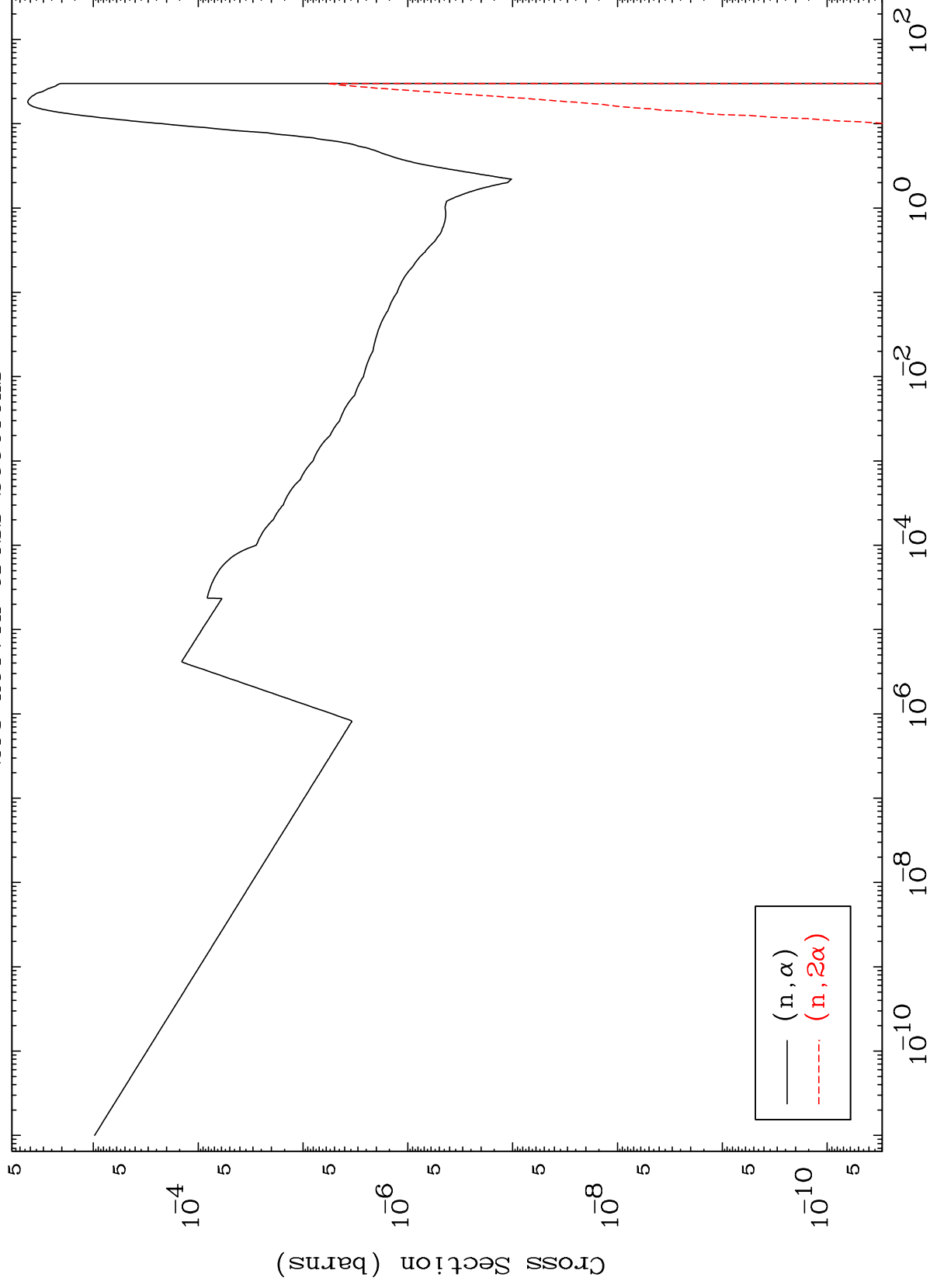
Incident Energy (MeV)

89-Ac-224

MAT 8922

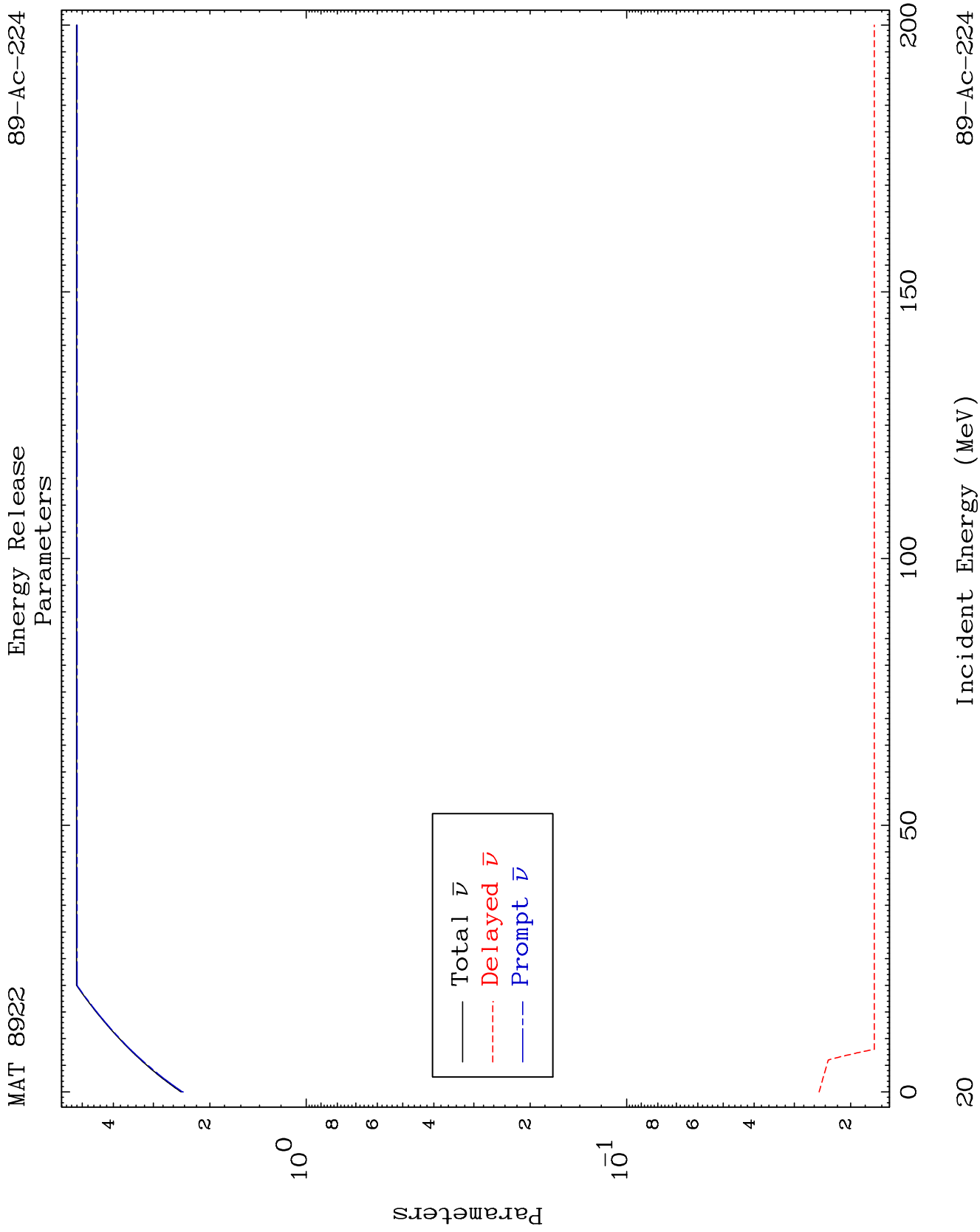
(n, α) Levels
293 Kelvin Cross Sections

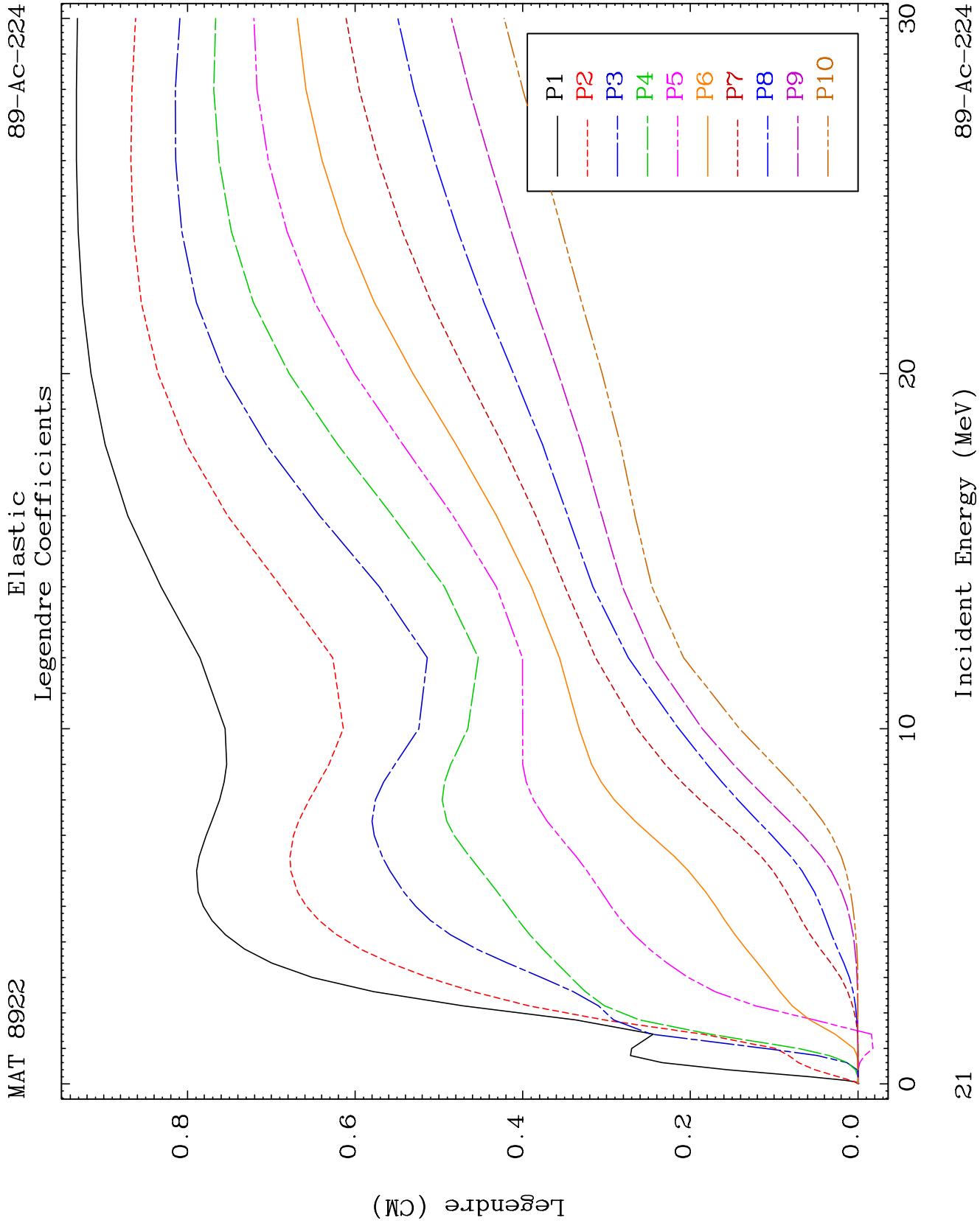
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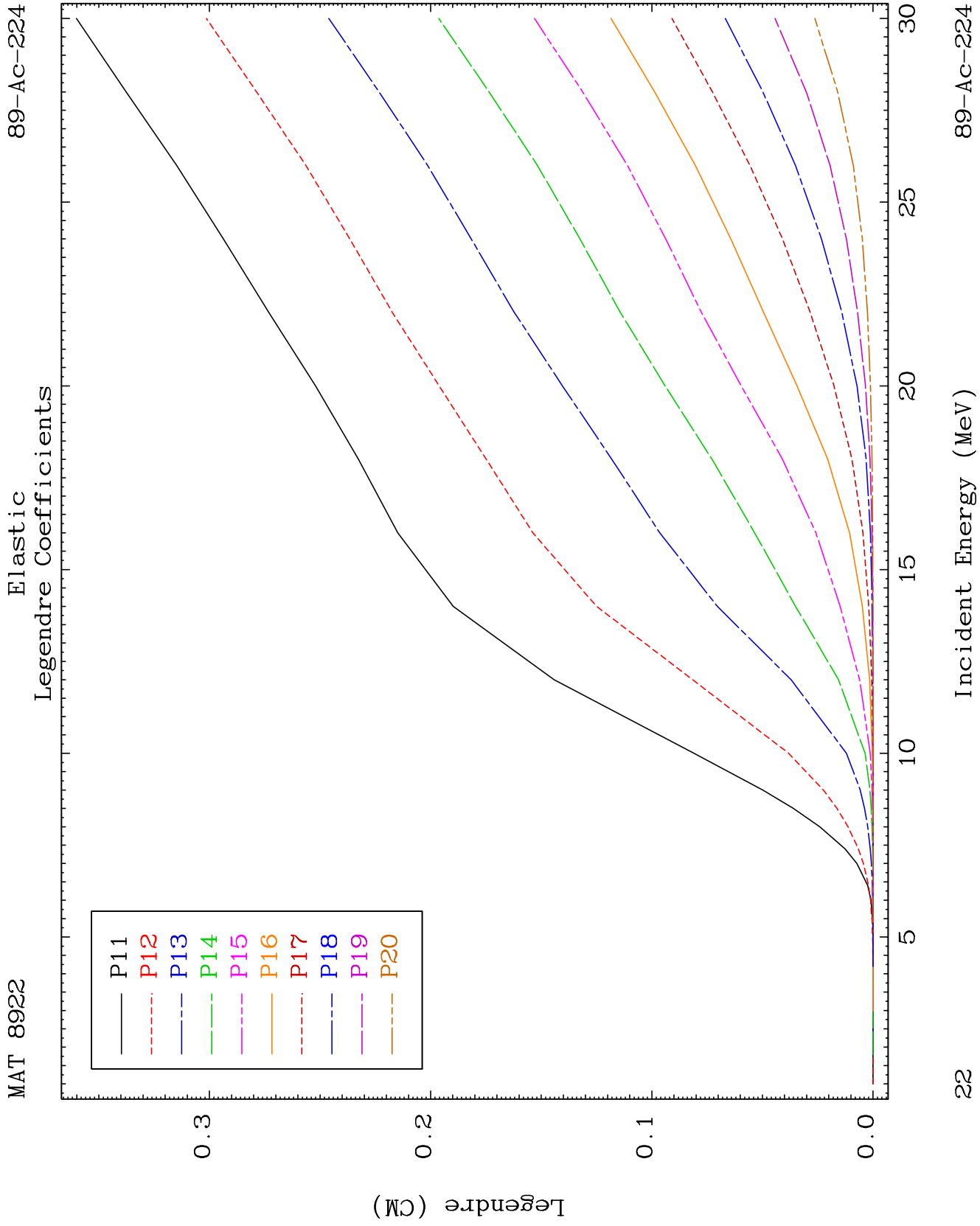


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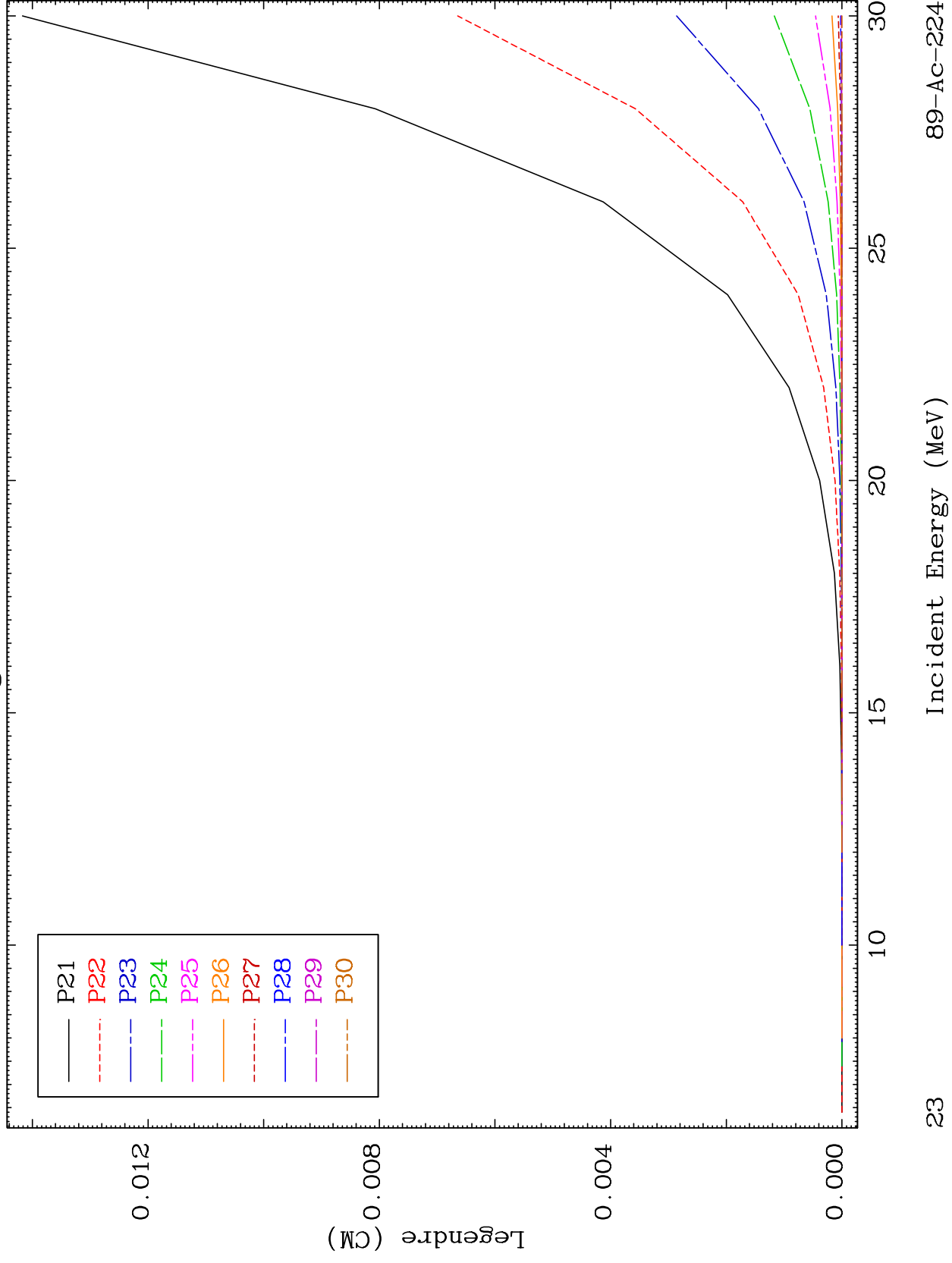




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Elastic Legendre Coefficients

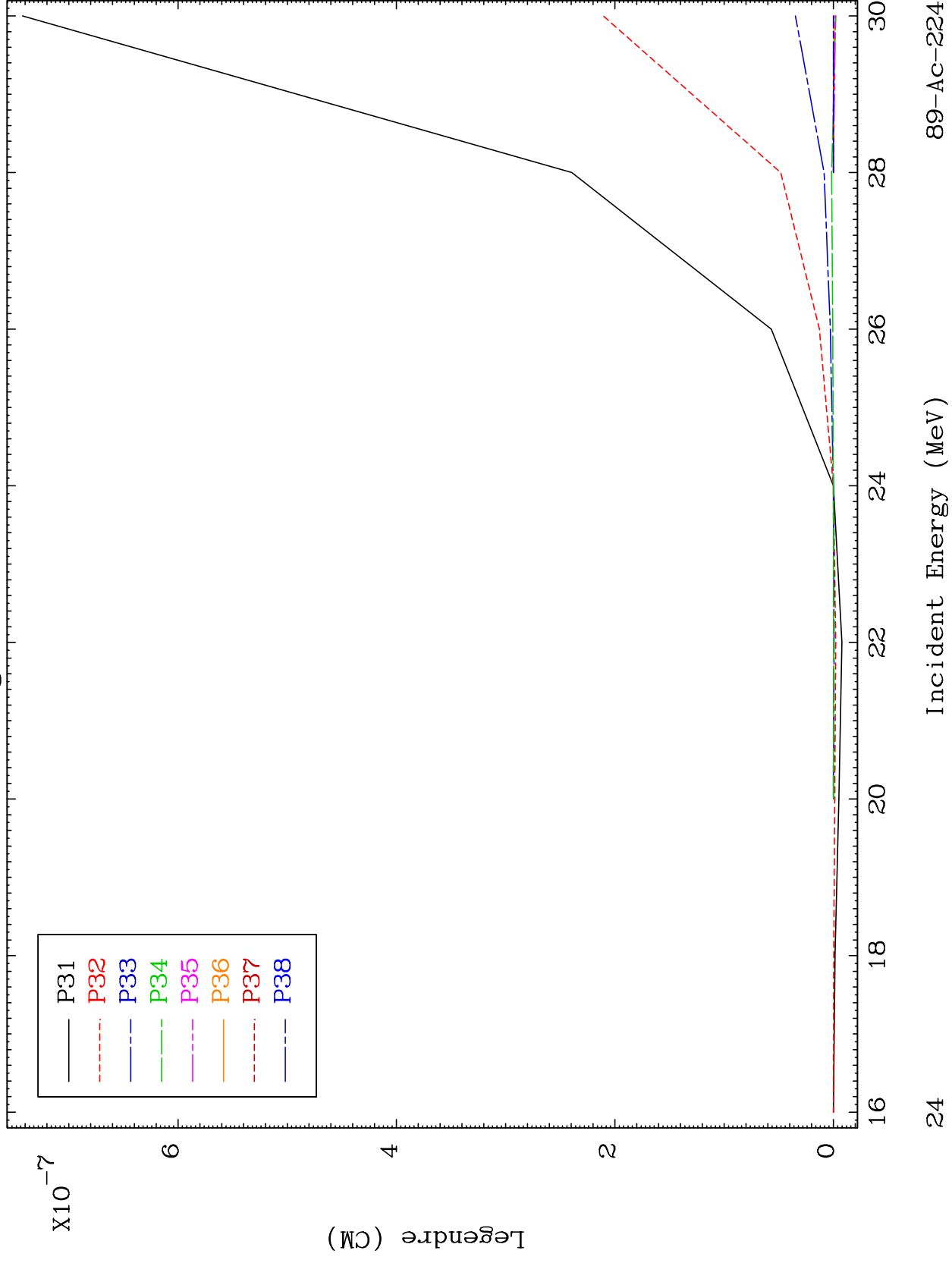
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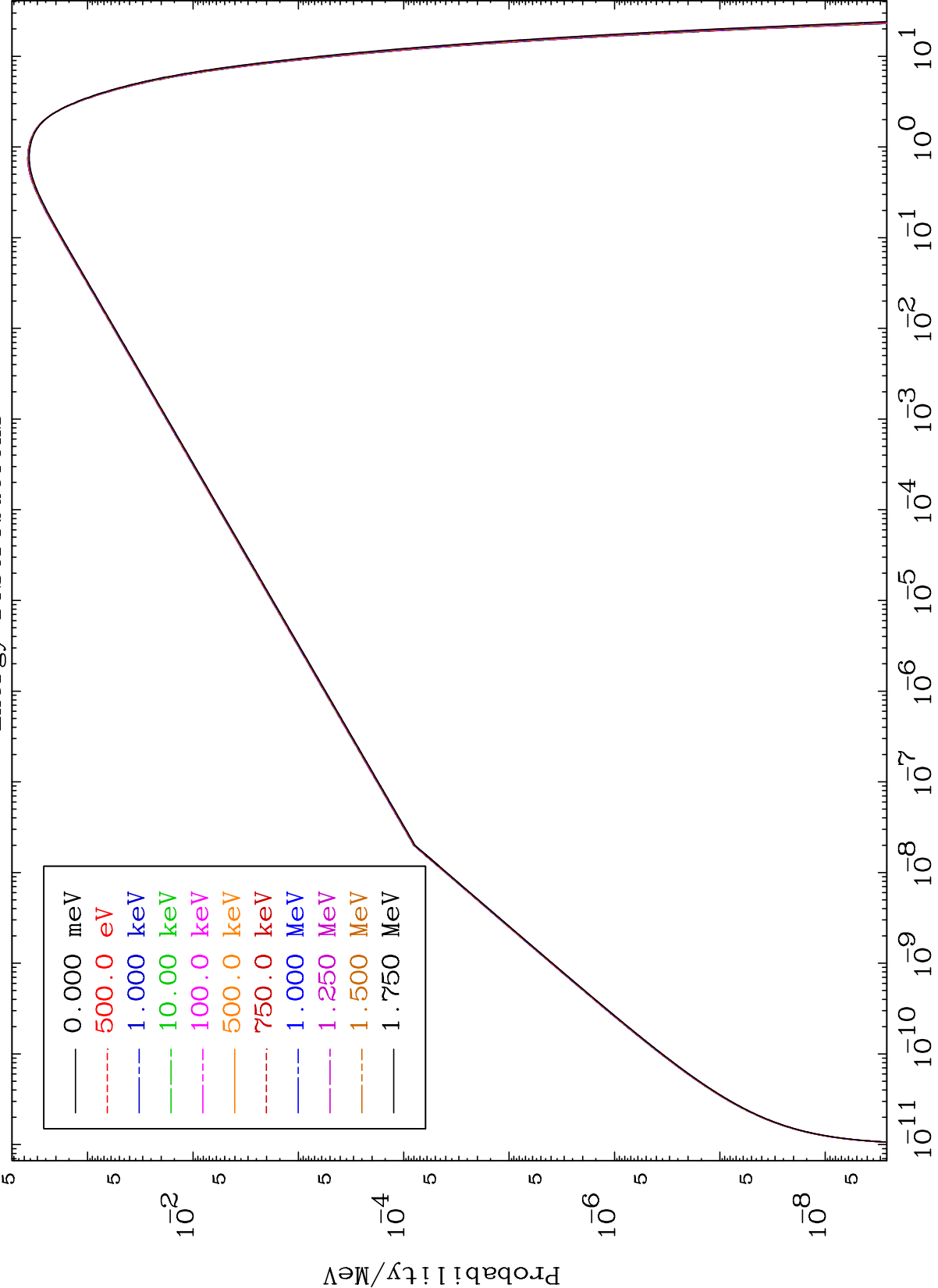
MAT 8922

Elastic Legendre Coefficients

89-AC-224



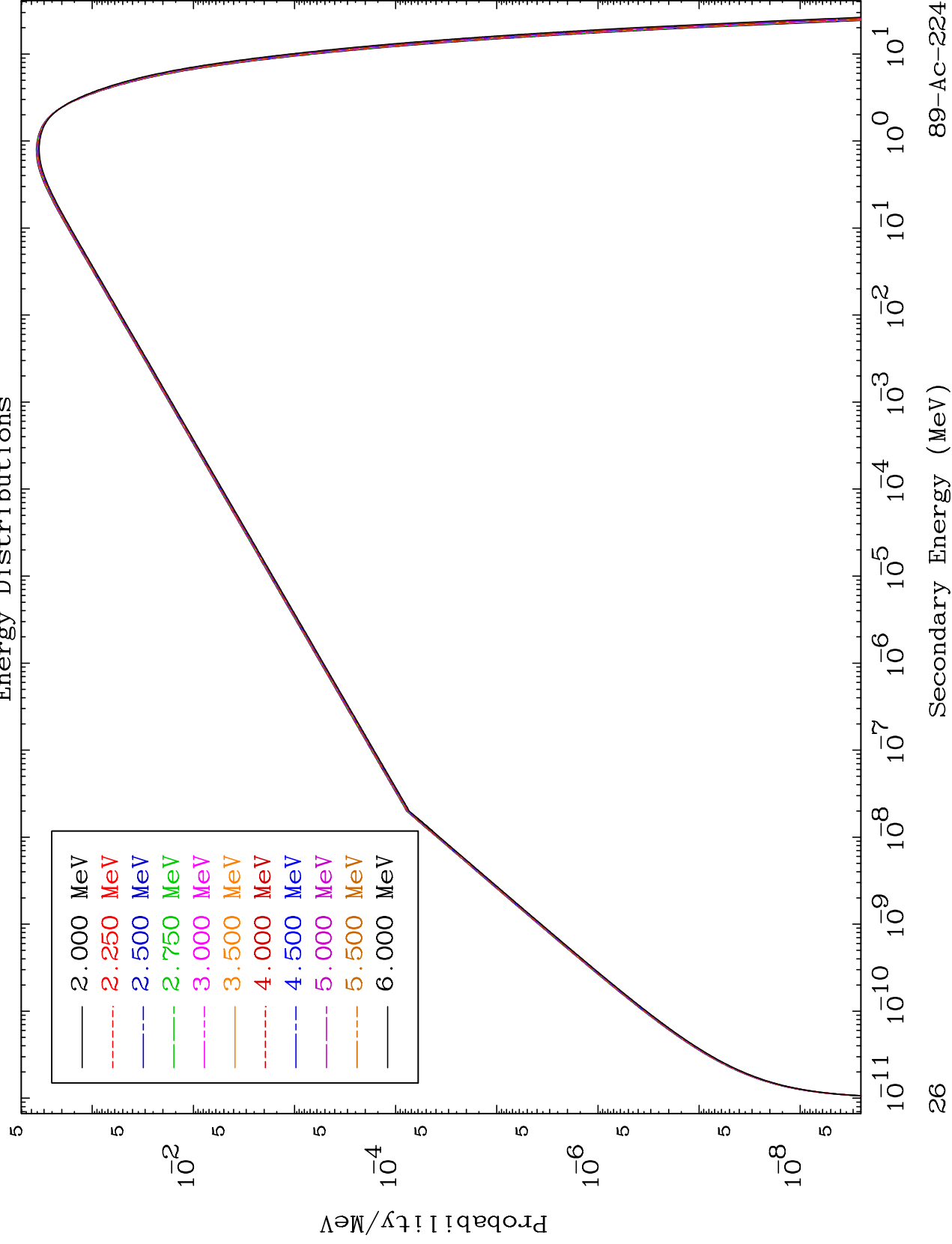
Fission
Energy Distributions

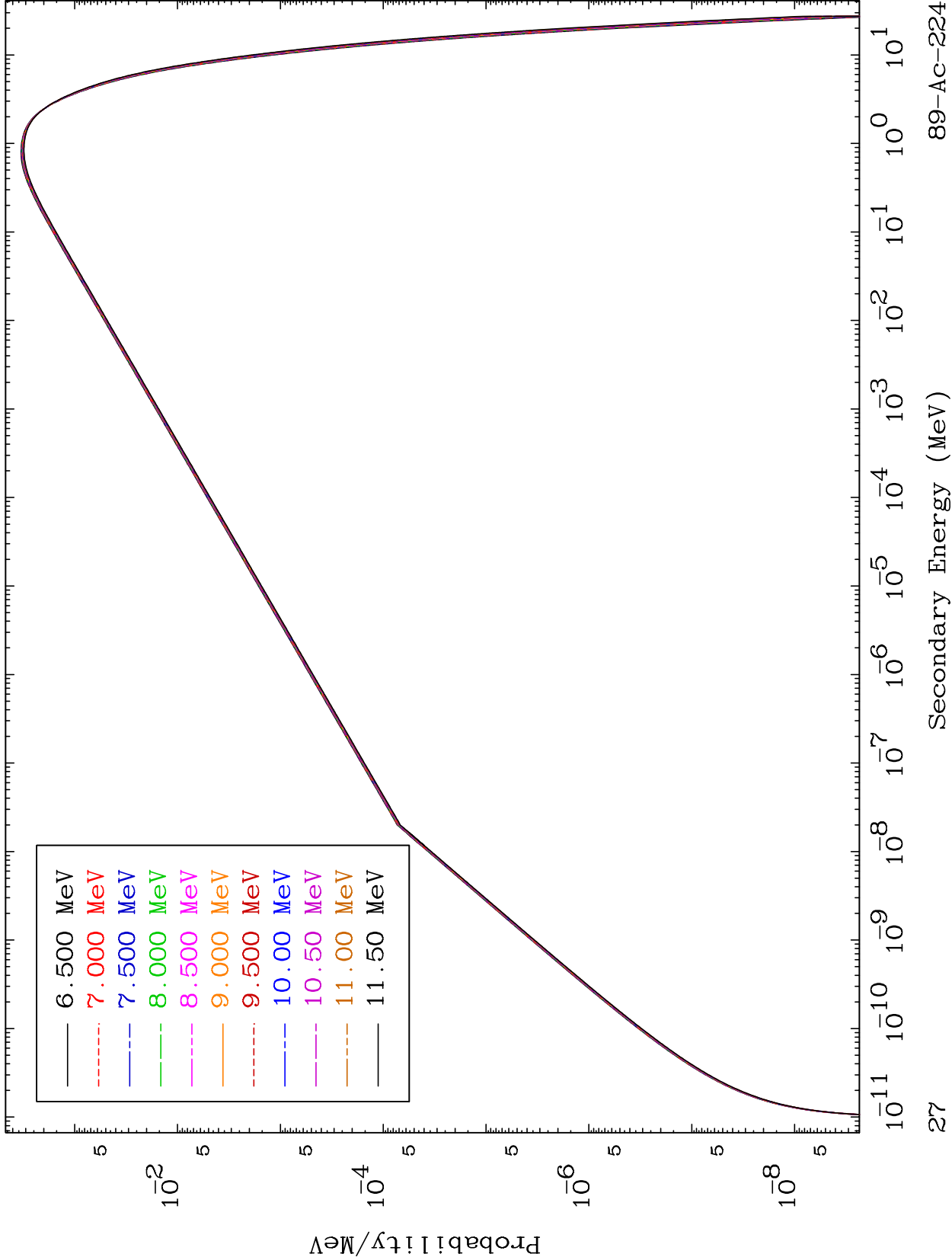


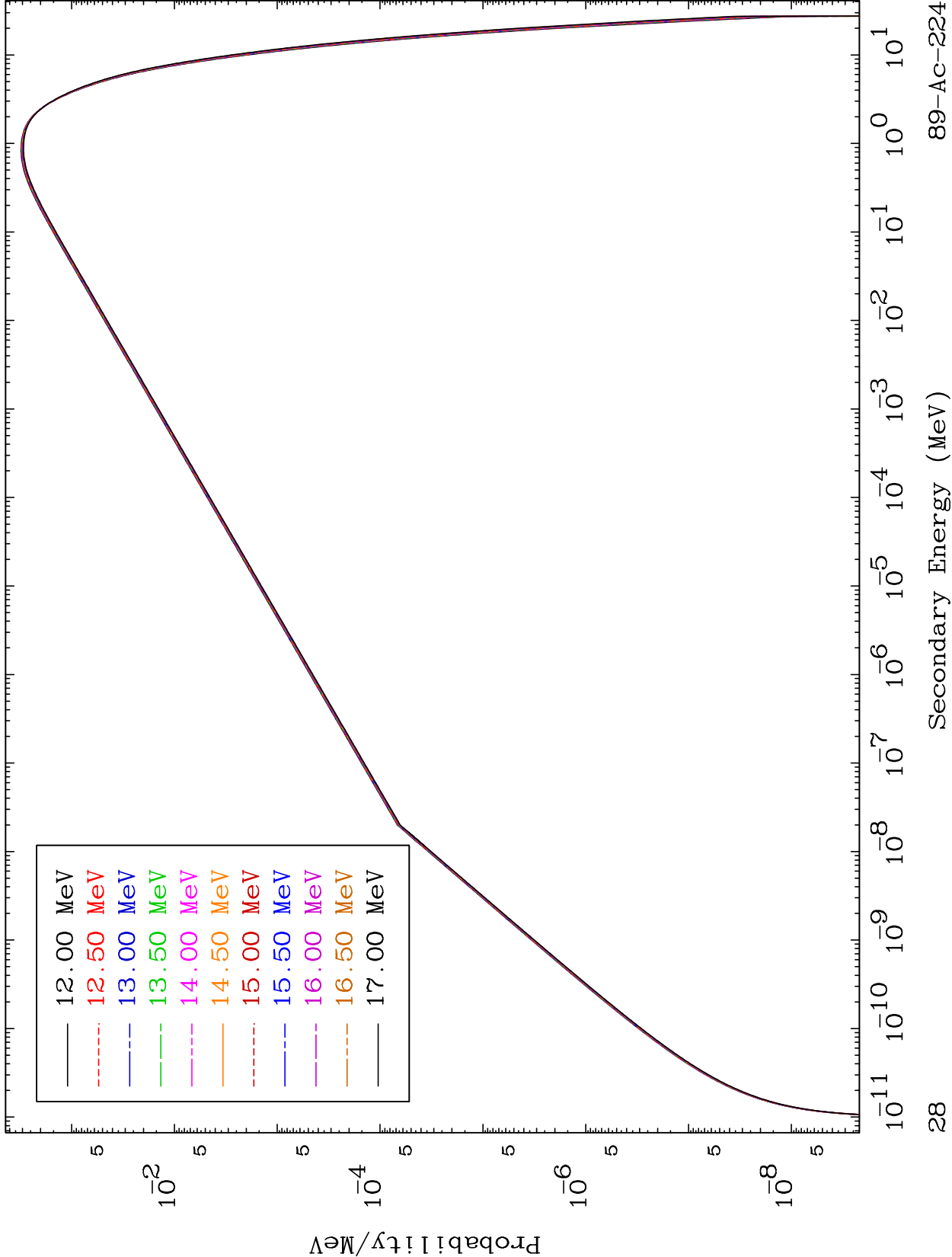
MAT 8922

Fission
Energy Distributions

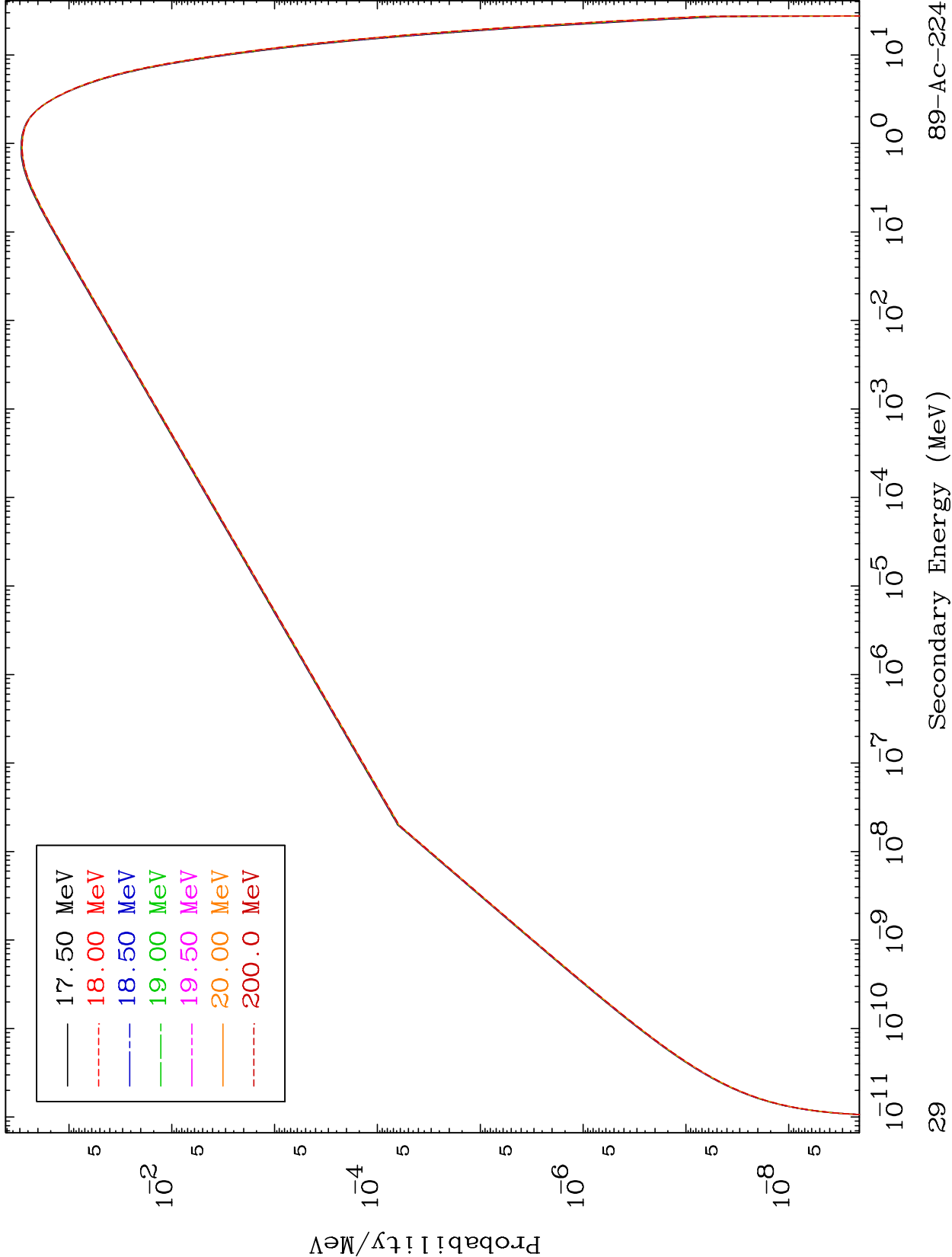
89-Ac-224







Fission
Energy Distributions



(n,3n)
Radionuclide Production Cross Section

