

Program EVALPLOT
(Version 2021-1)

by

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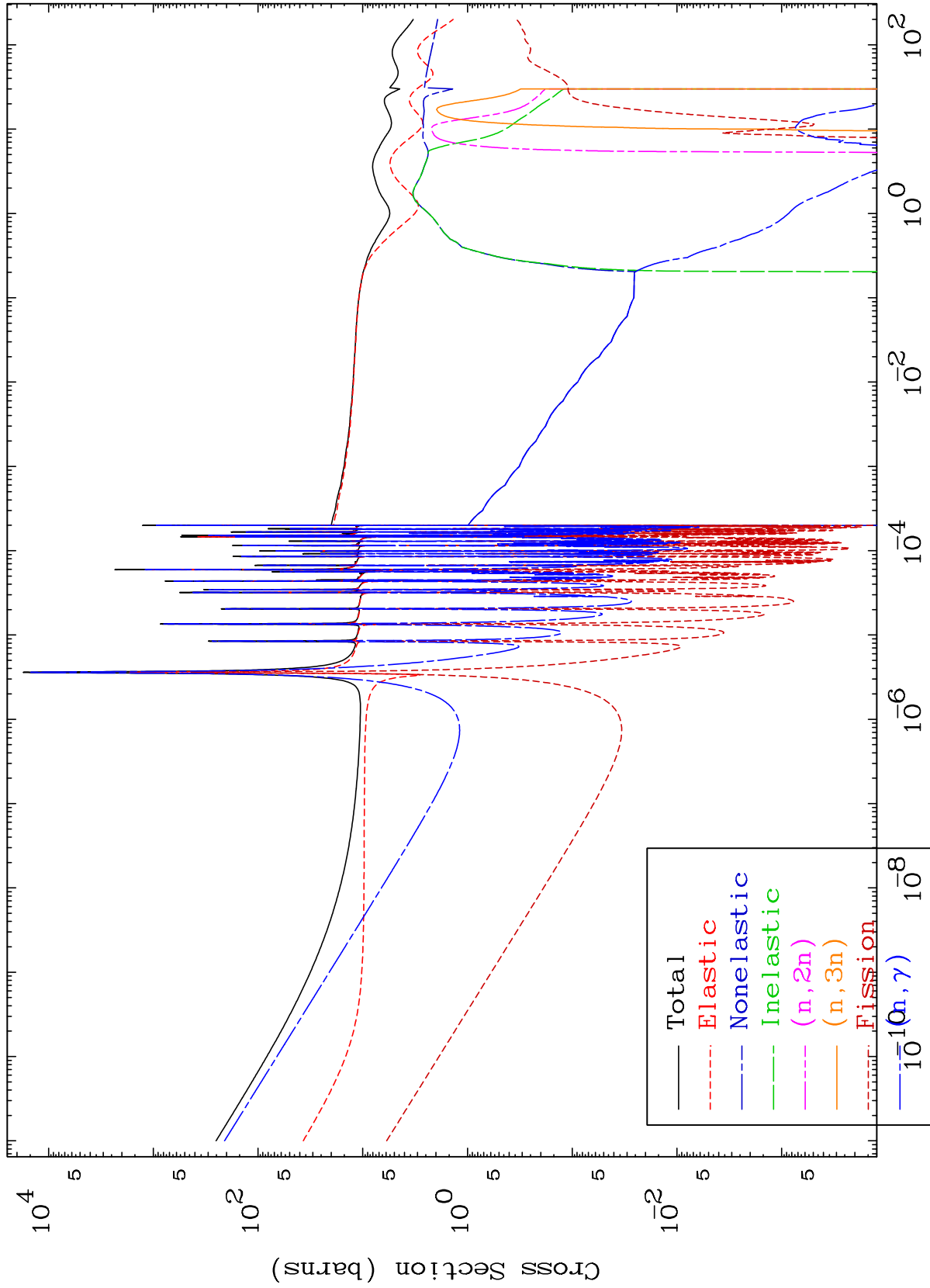
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8961

Neutron Major
293 Kelvin Cross Sections

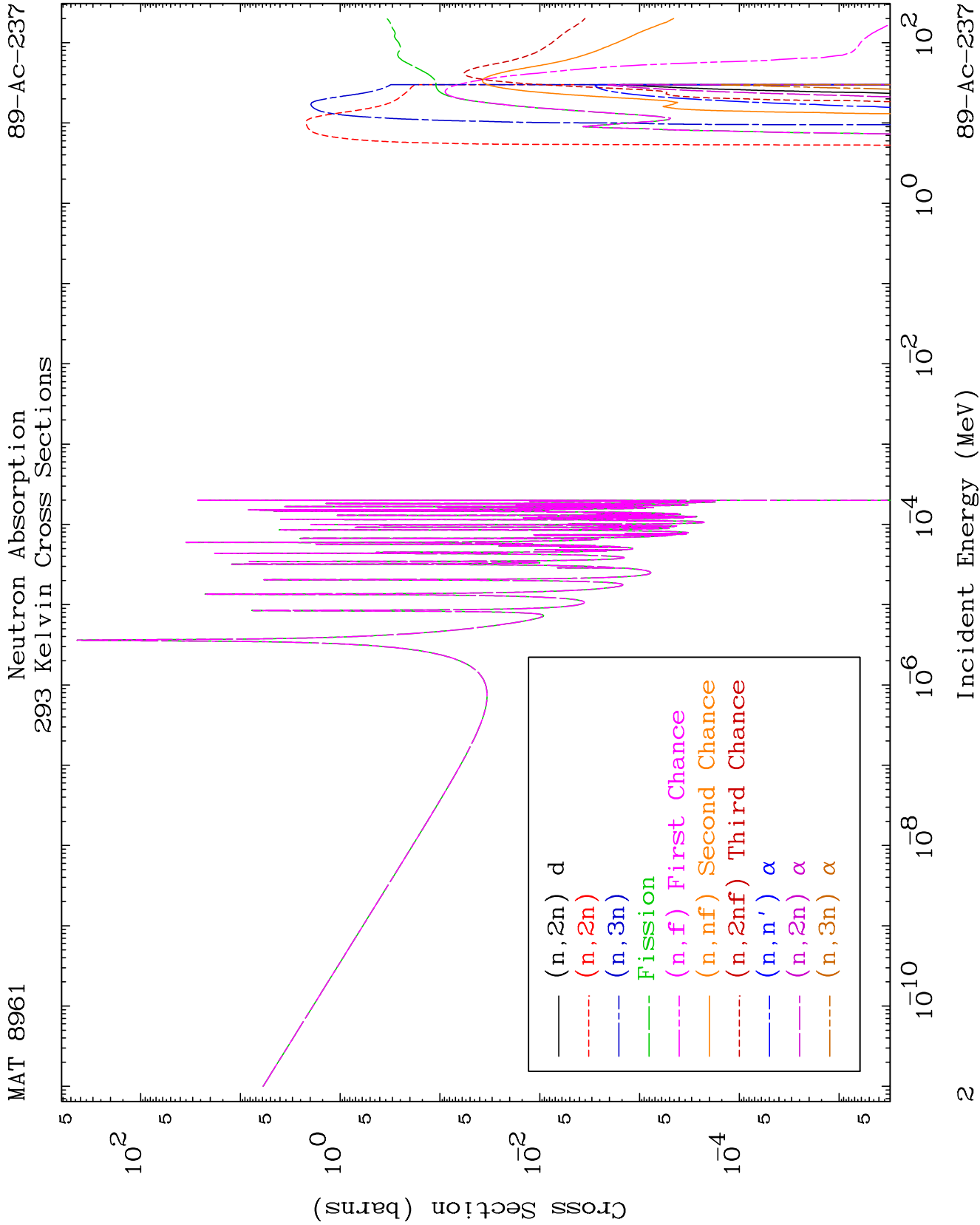
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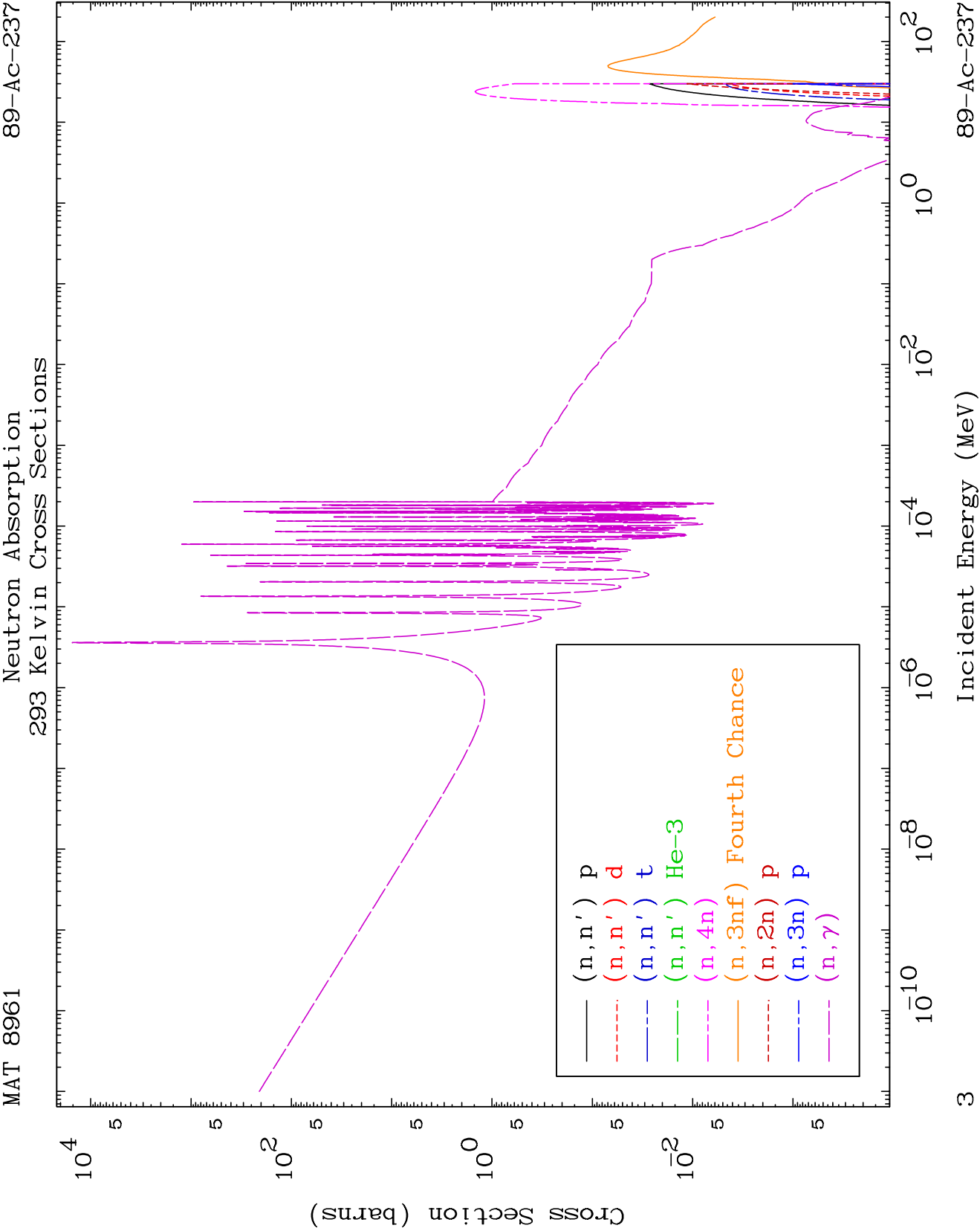


1

Incident Energy (MeV)

89-Ac-237

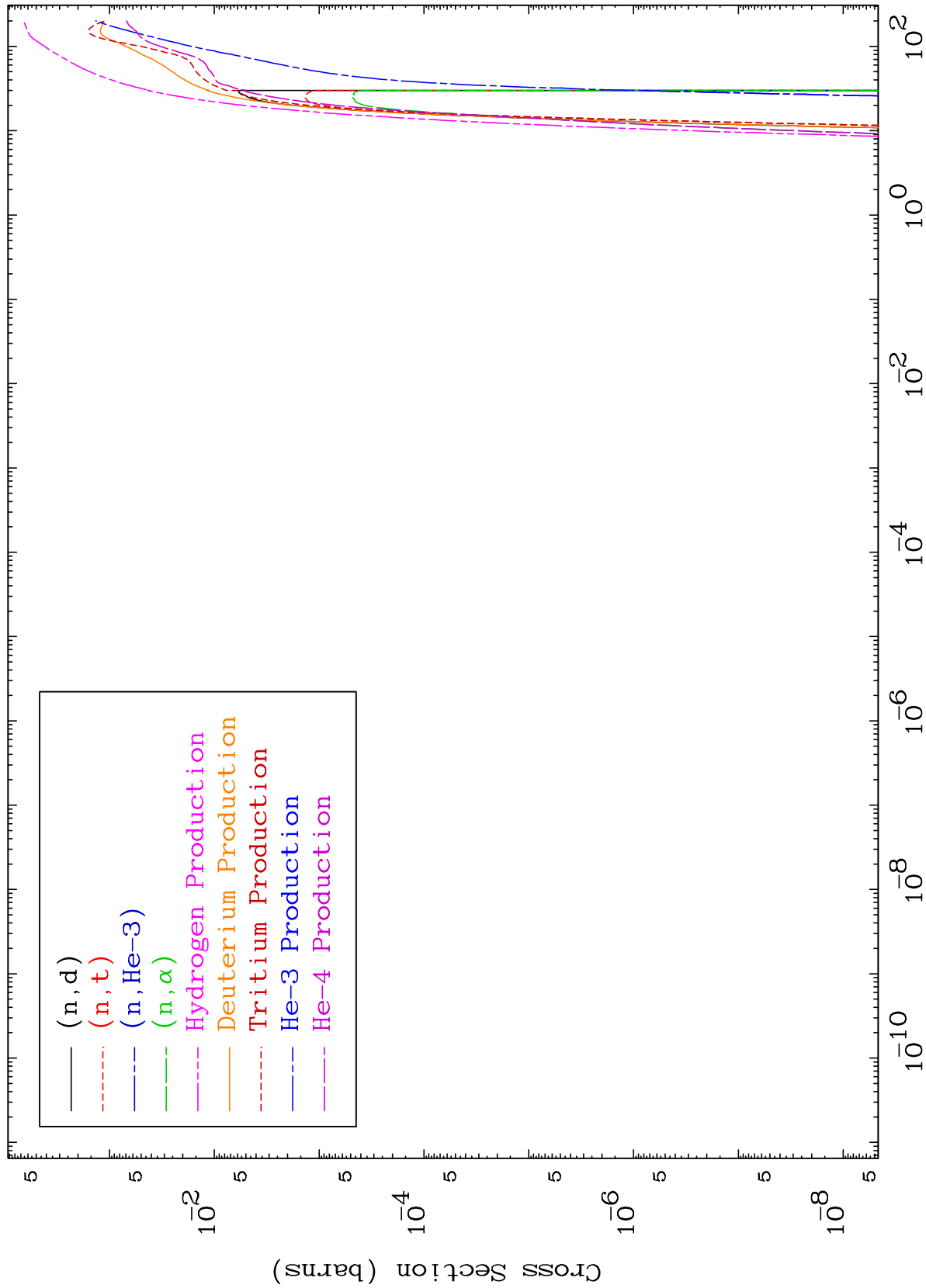




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Neutron Absorption
293 Kelvin Cross Sections

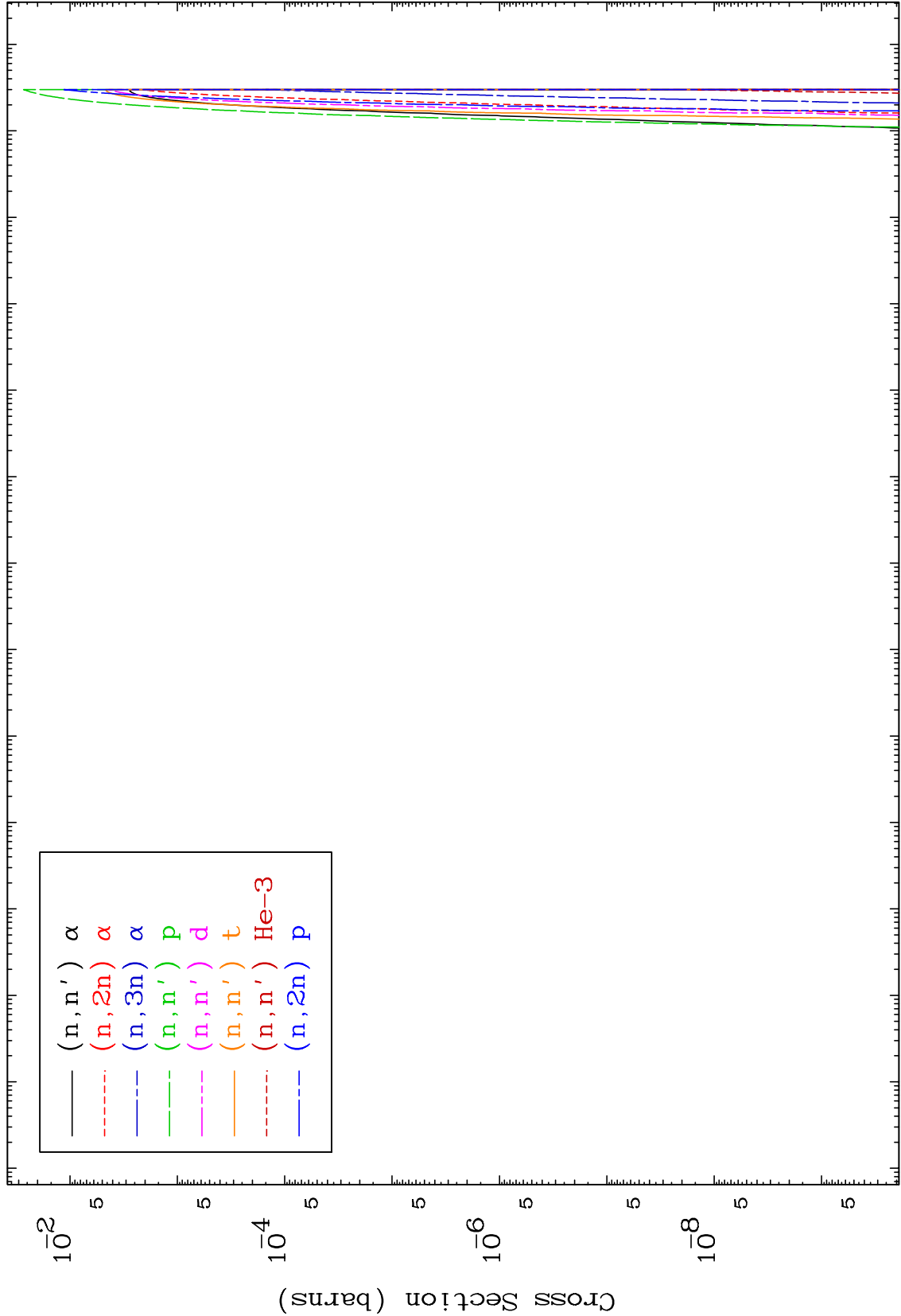
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Charged Particle
293 Kelvin Cross Sections

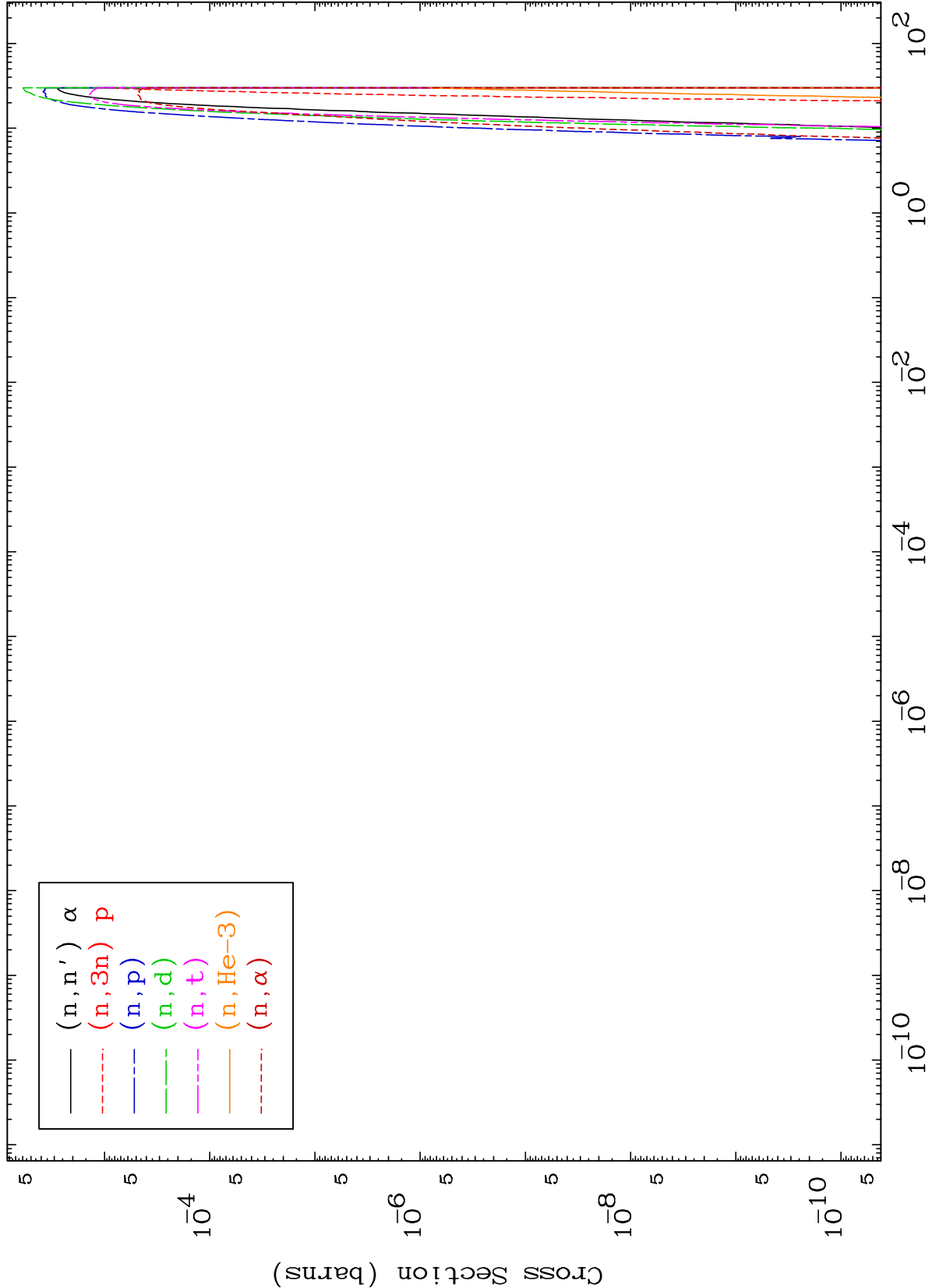
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5

Incident Energy (MeV)

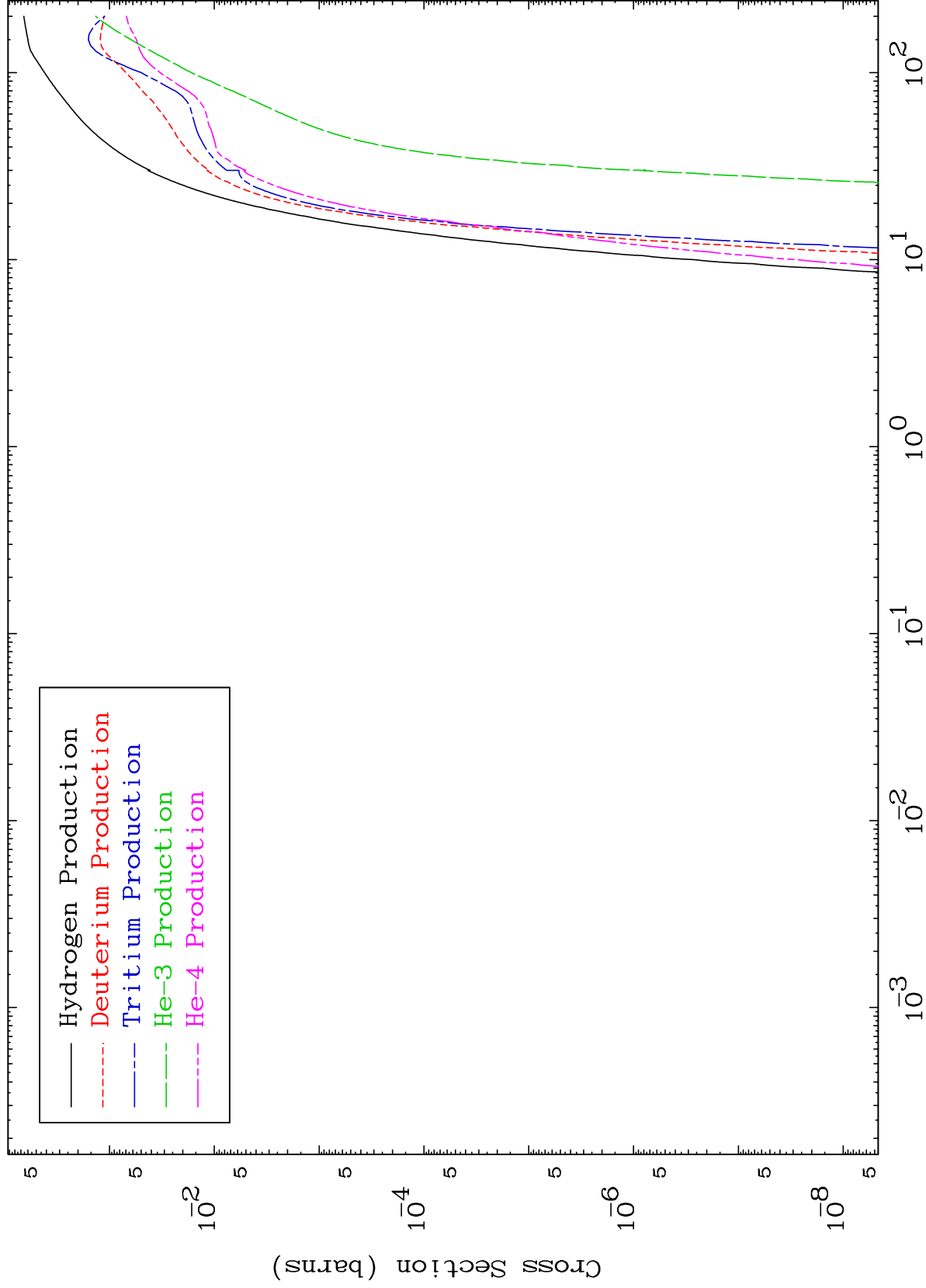
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Particle Production
293 Kelvin Cross Sections

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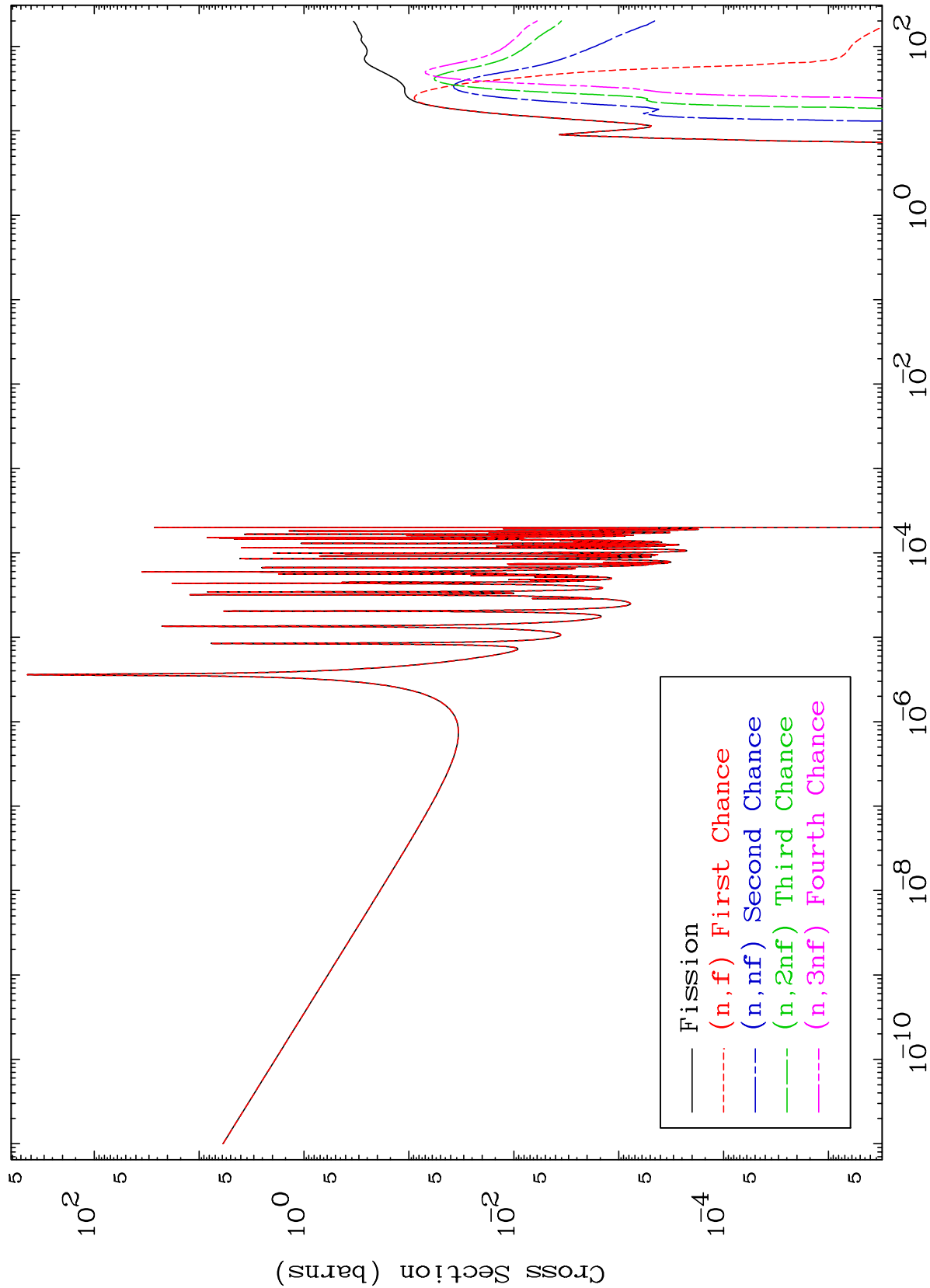


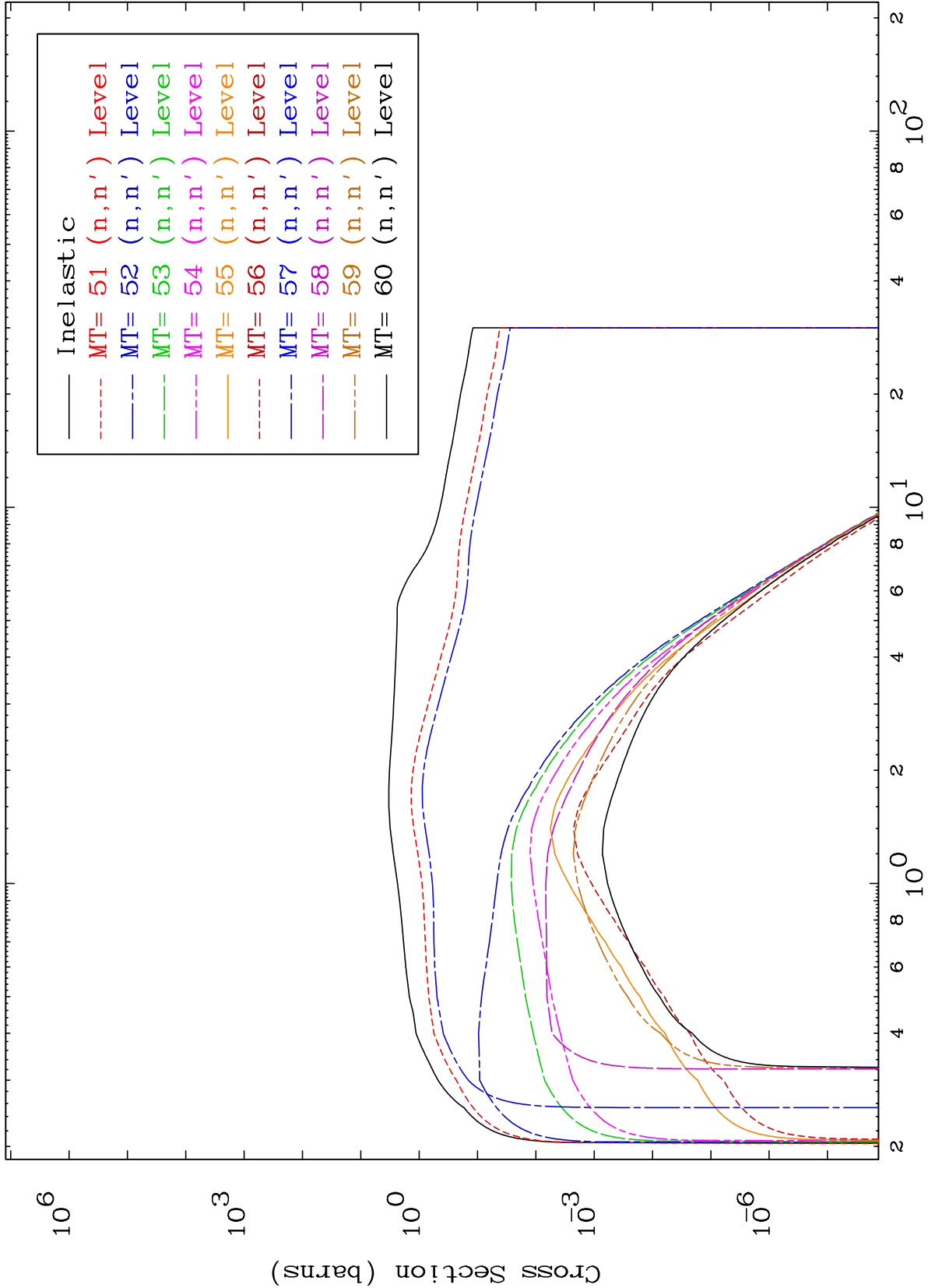
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Fission

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293 Kelvin Cross Sections



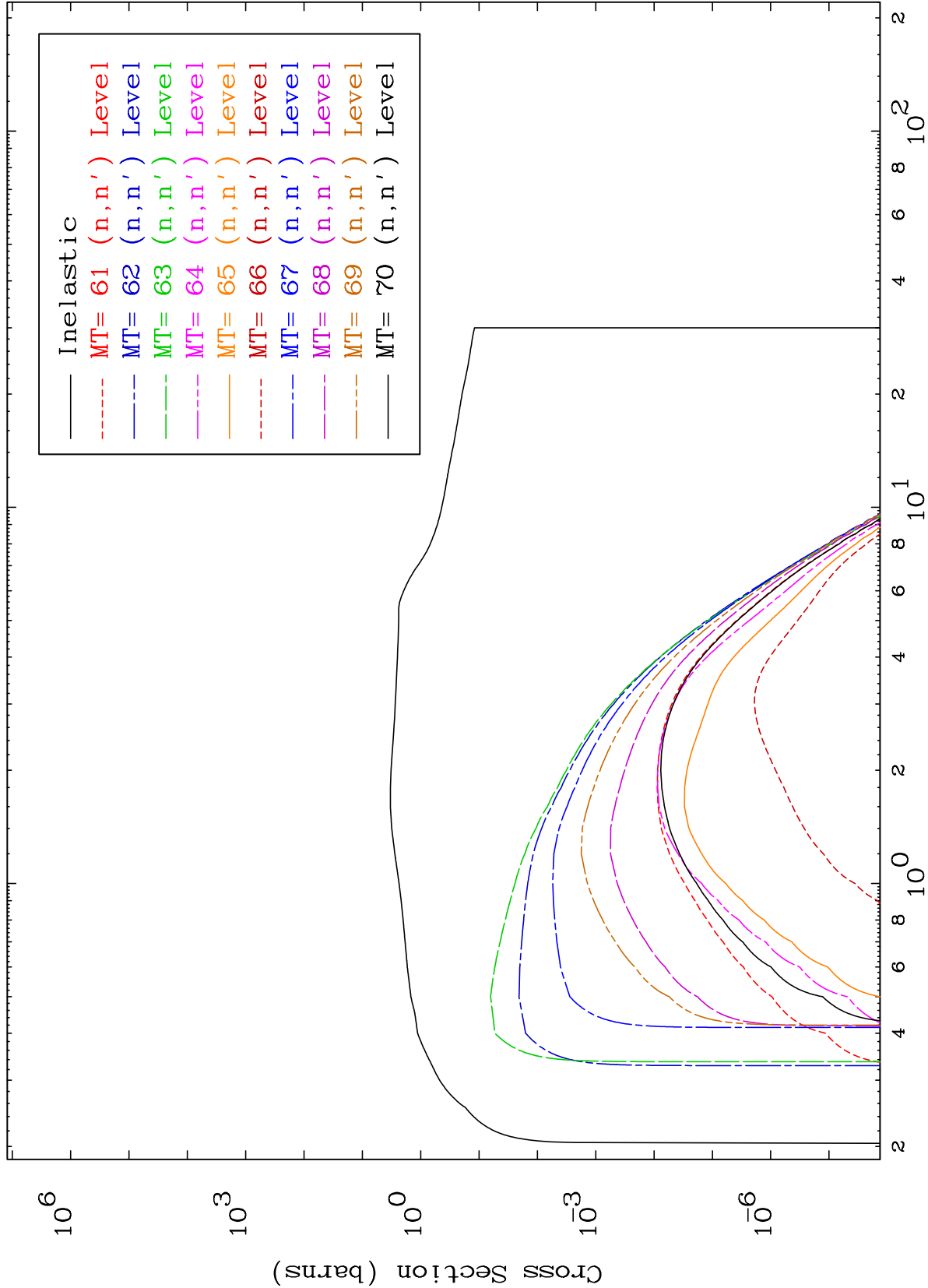


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(n,n') Levels

293 Kelvin Cross Sections

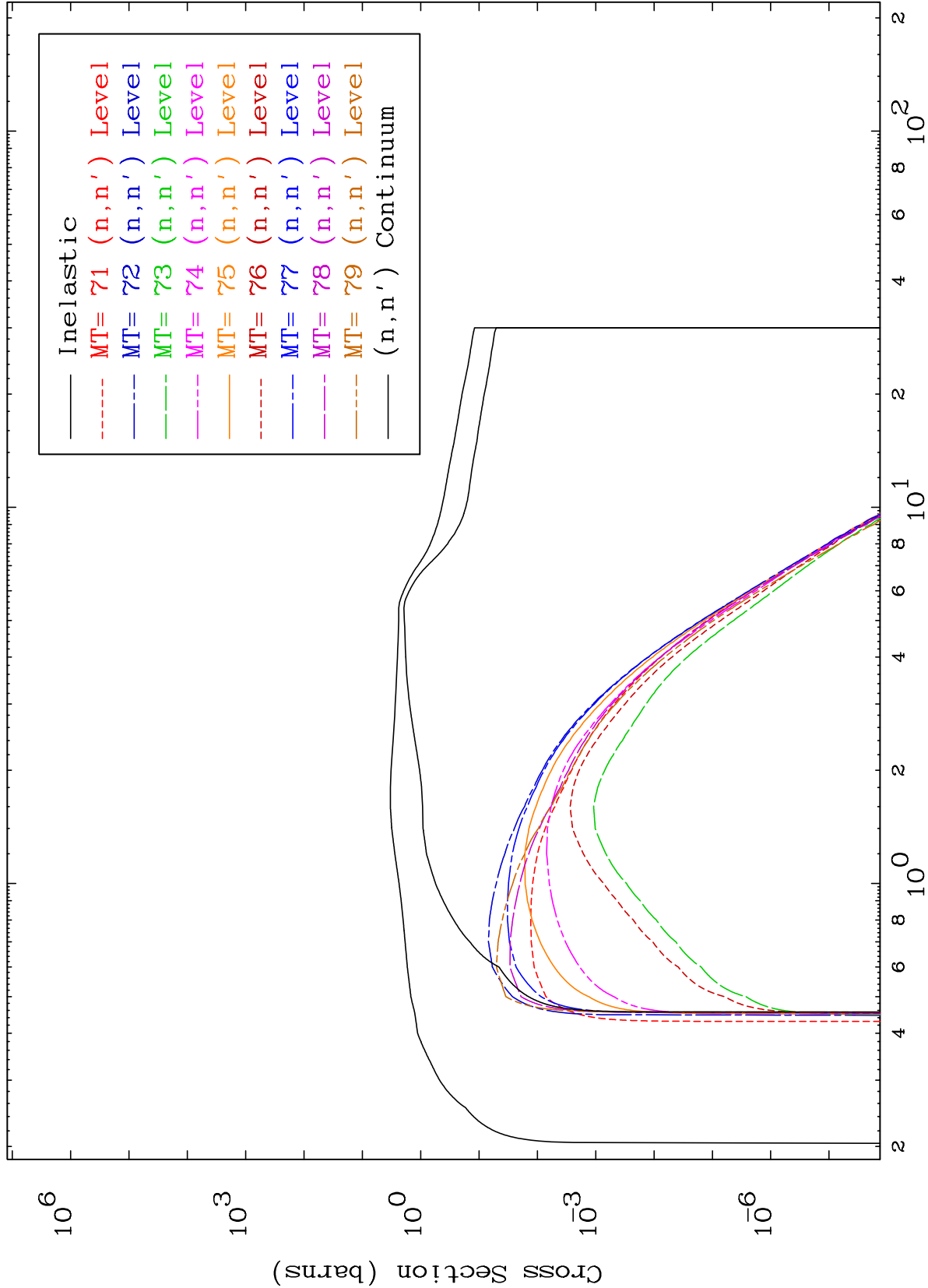
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10

Incident Energy (MeV)

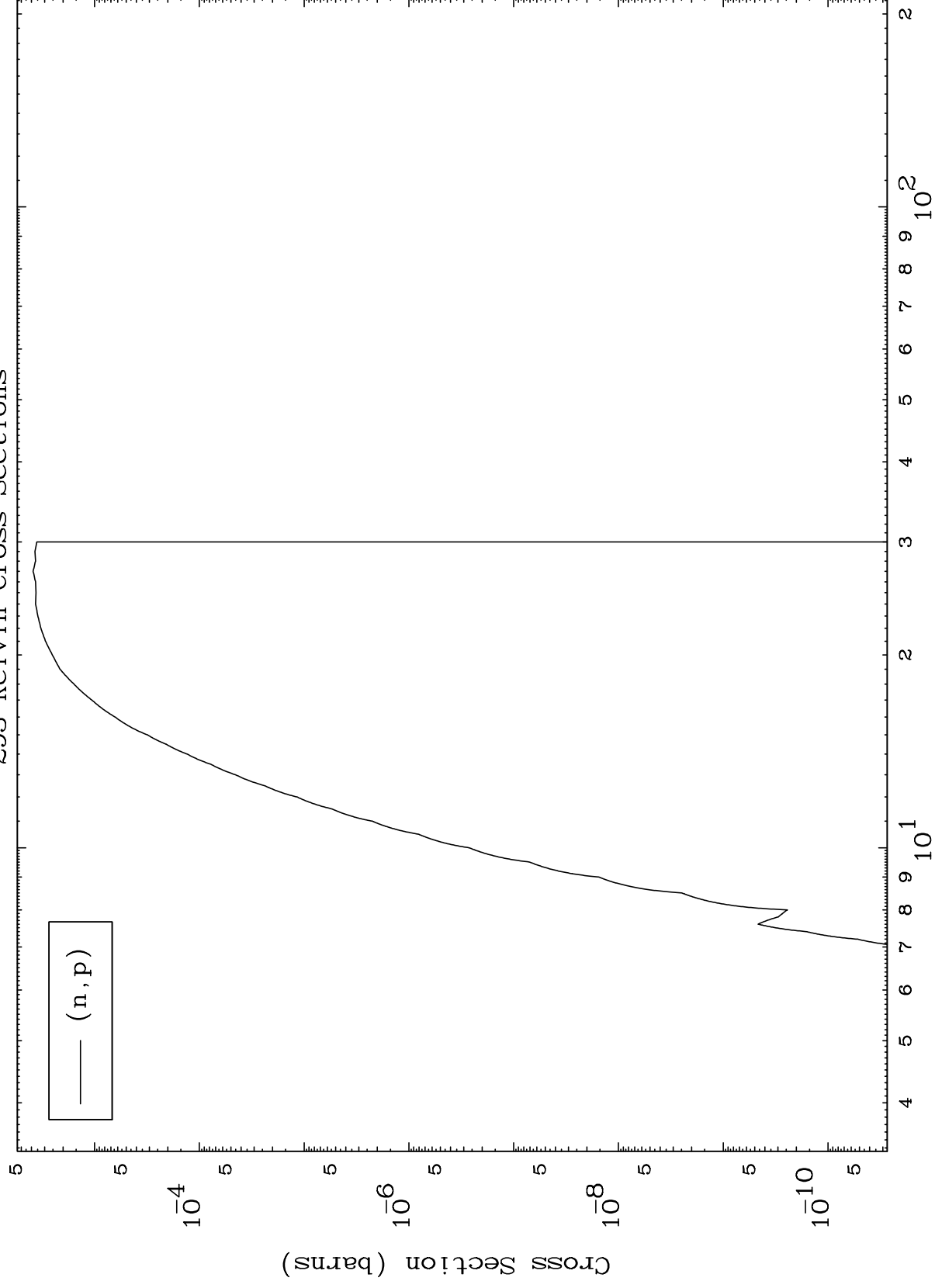
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(n,p) Levels
293 Kelvin Cross Sections



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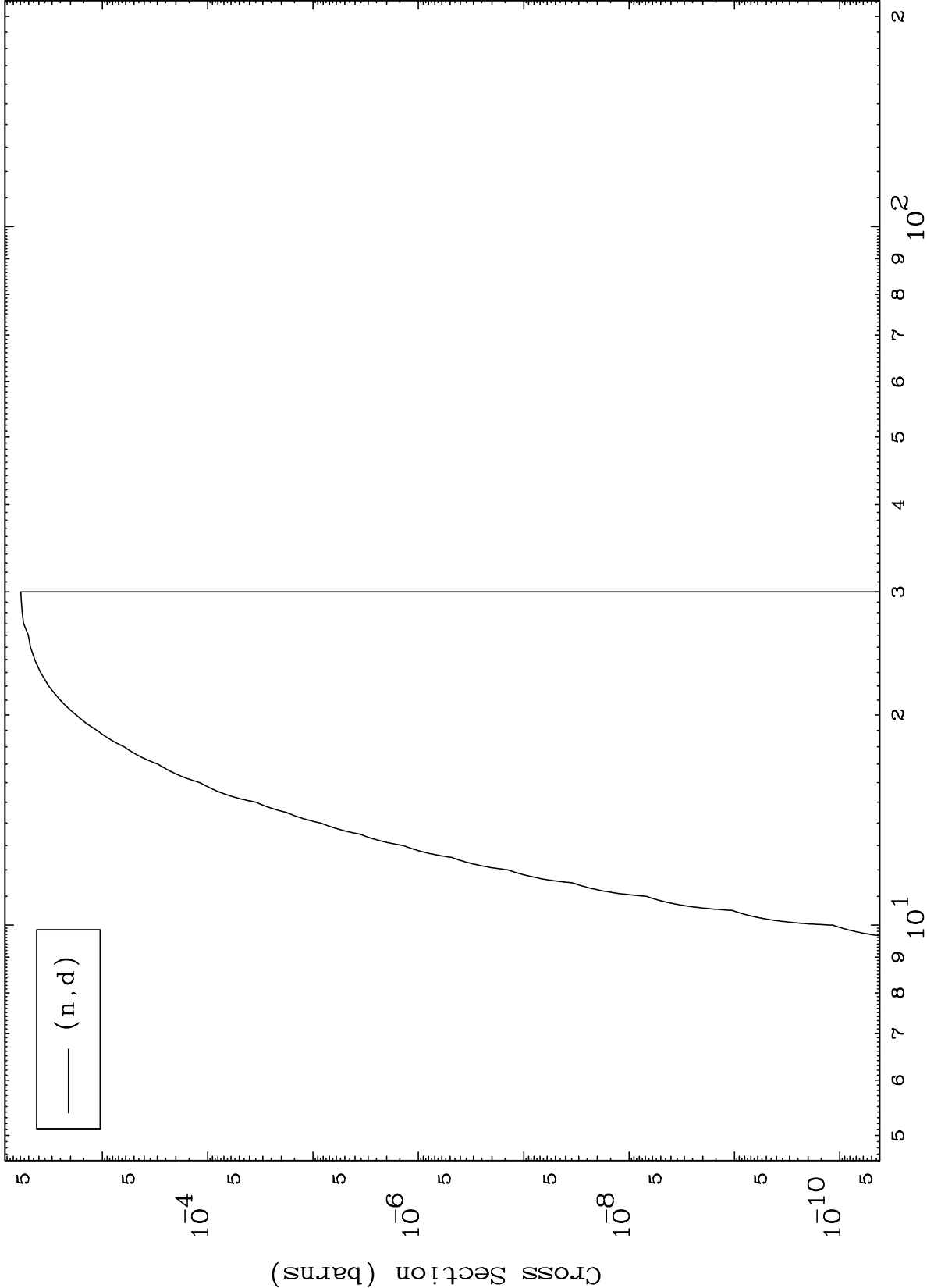
Incident Energy (MeV)

12

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(n,d) Levels
293 Kelvin Cross Sections

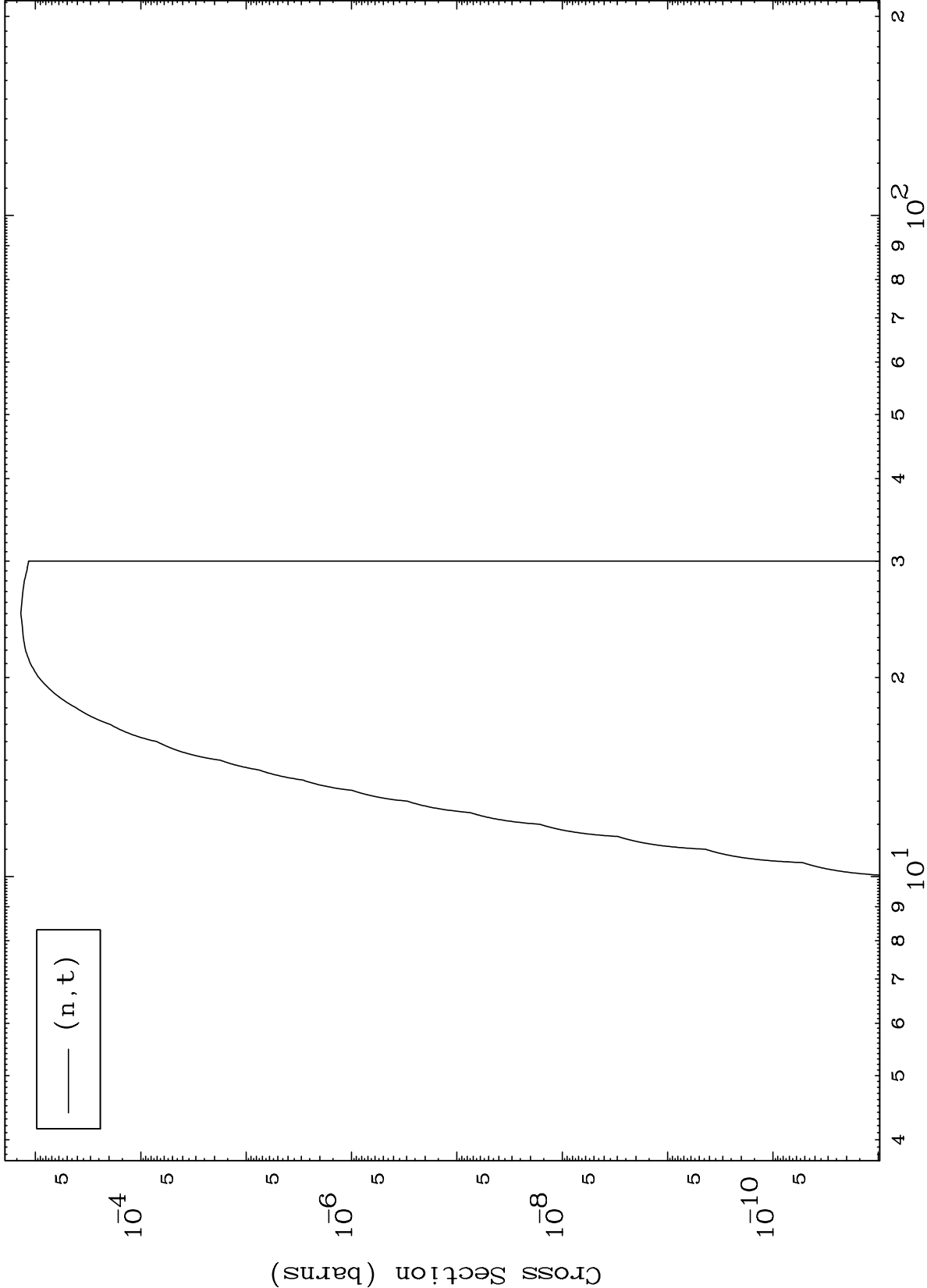
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(n,t) Levels
293 Kelvin Cross Sections

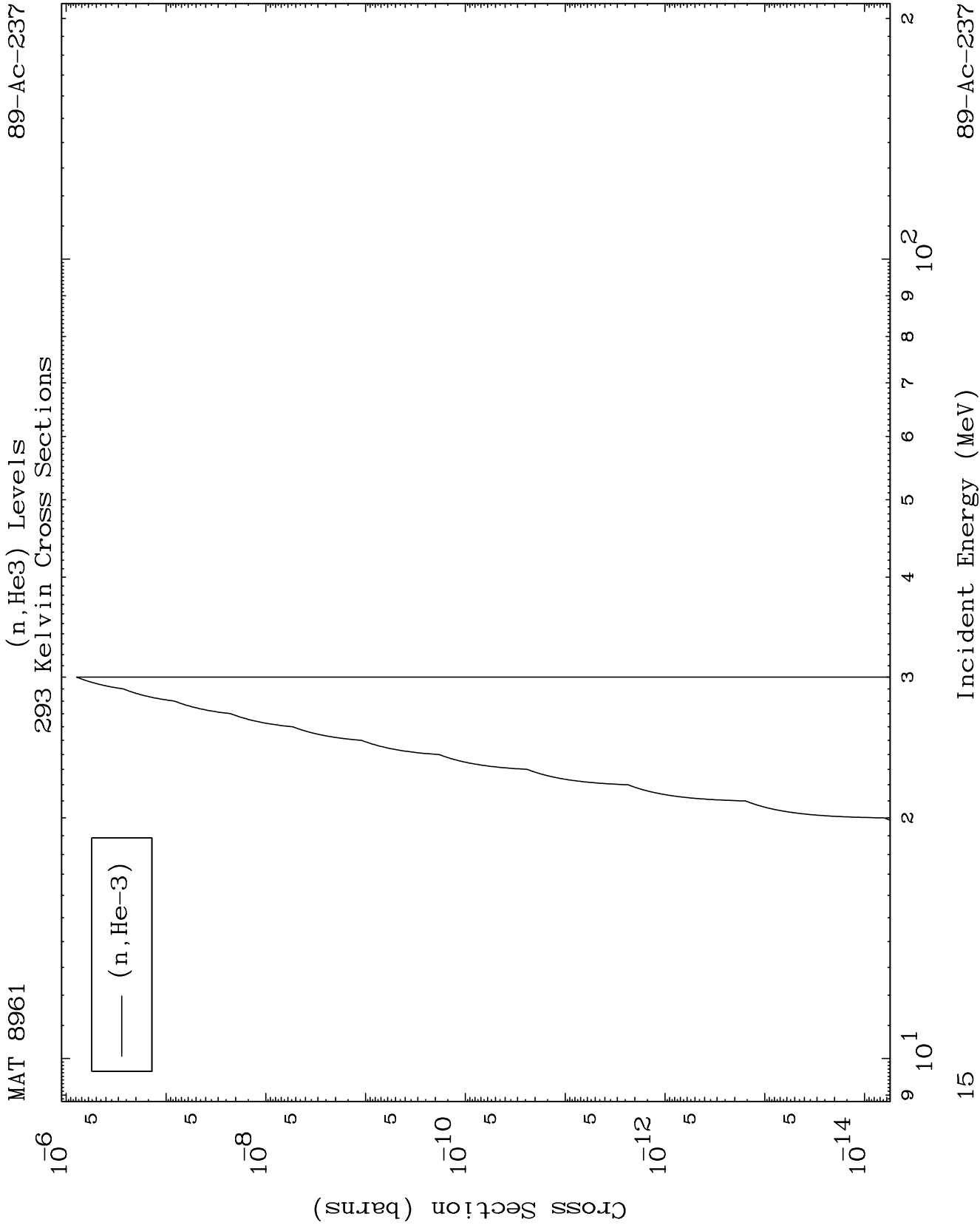
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Incident Energy (MeV)

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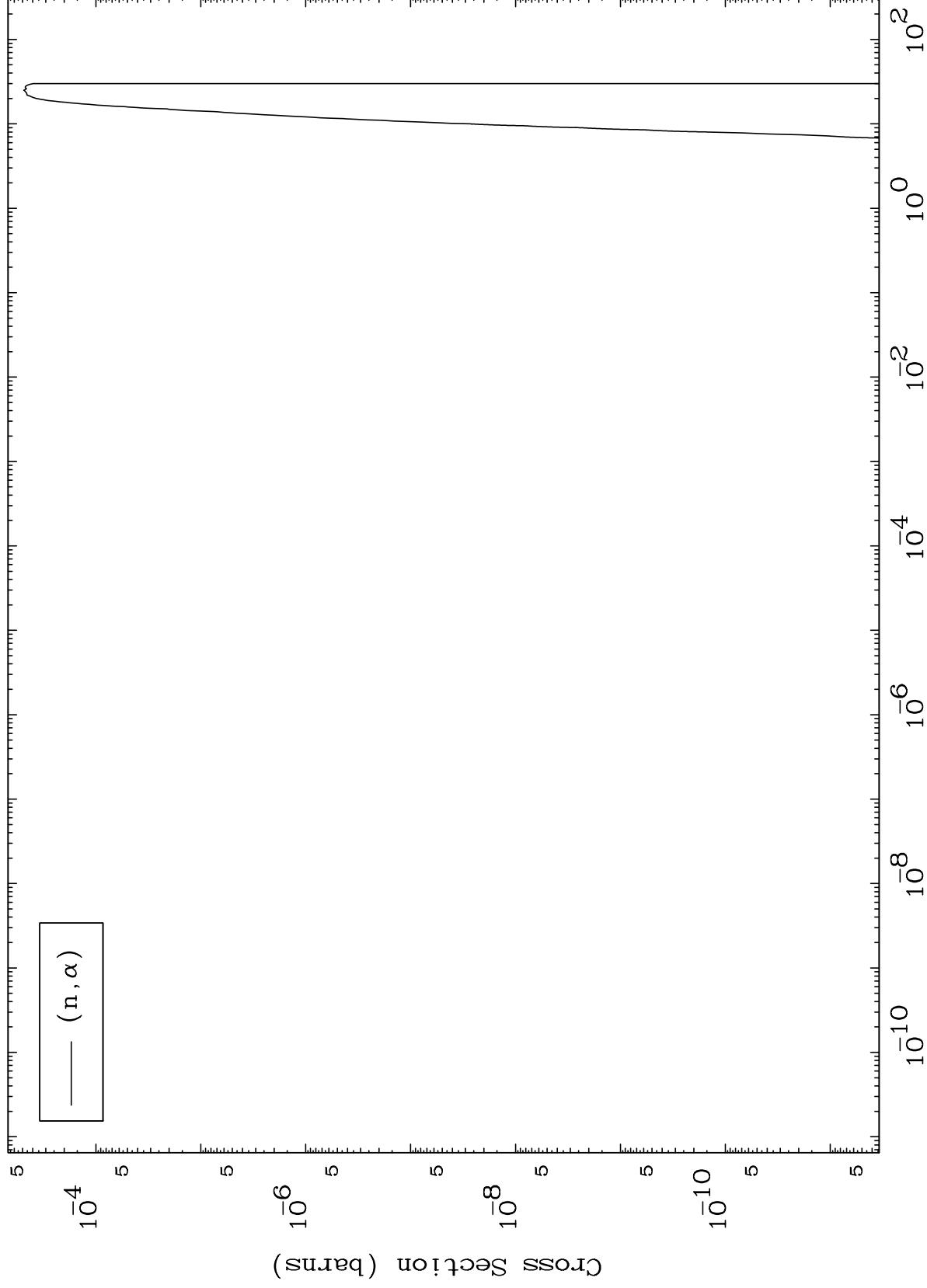
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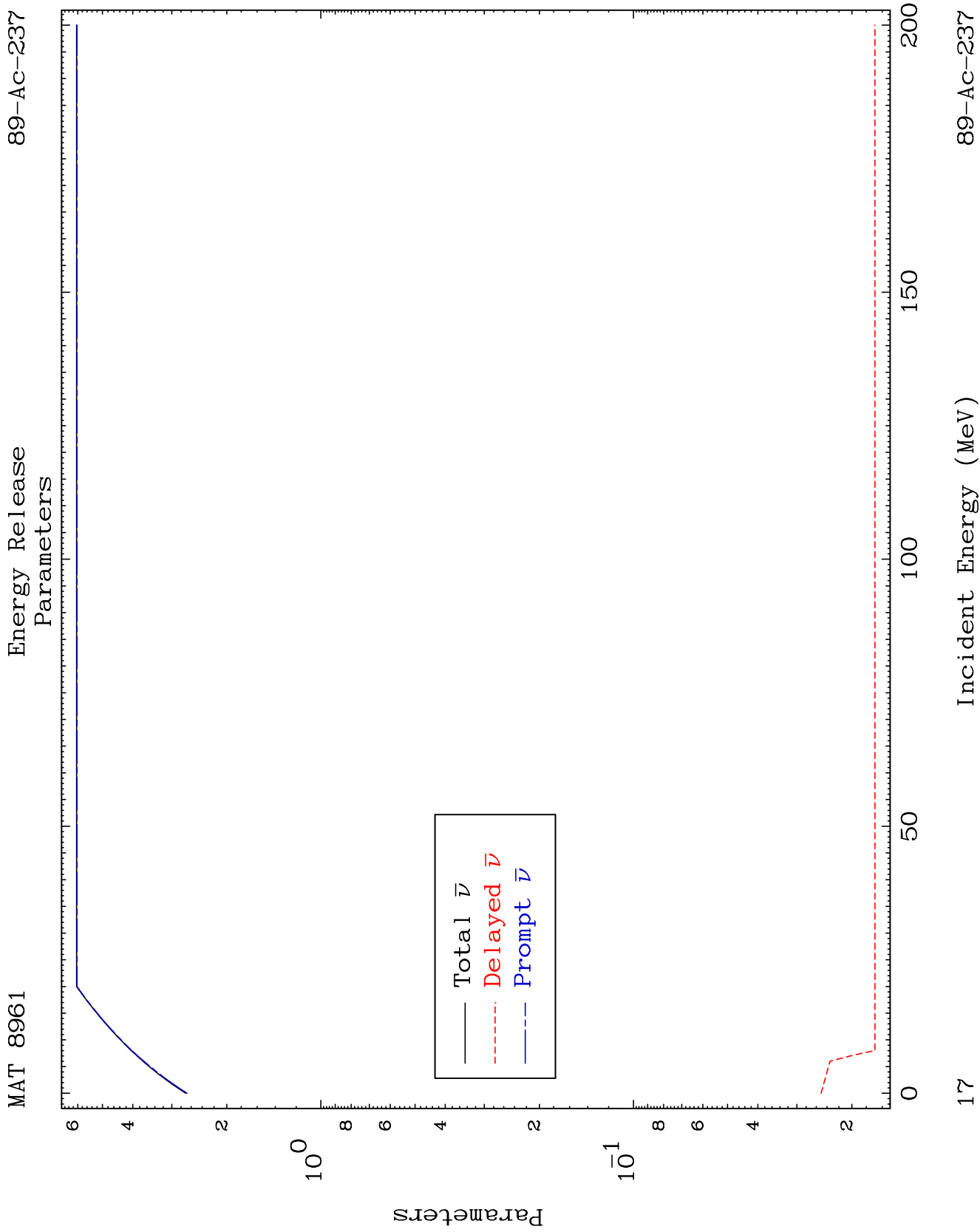
(n, α) Levels
293 Kelvin Cross Sections

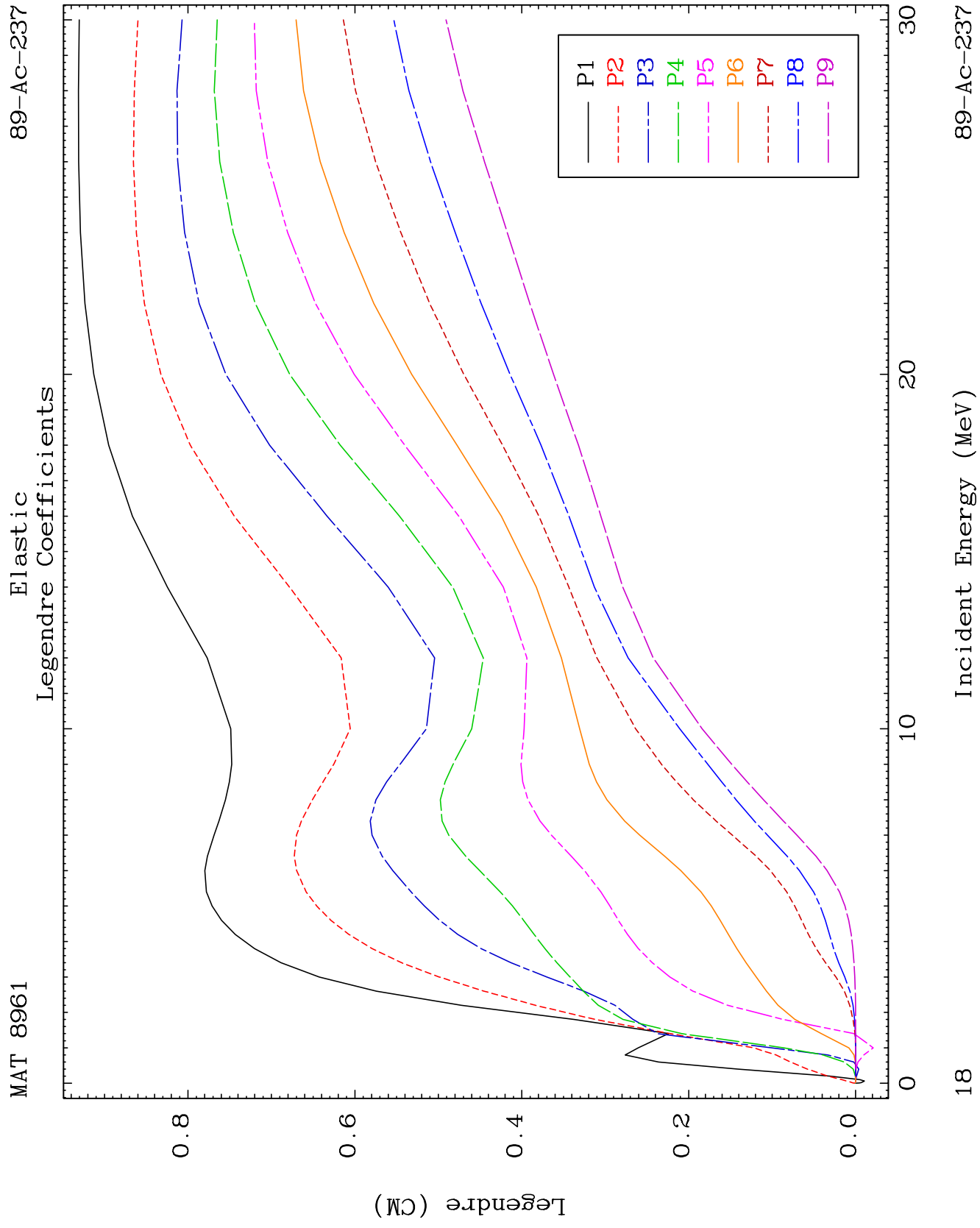
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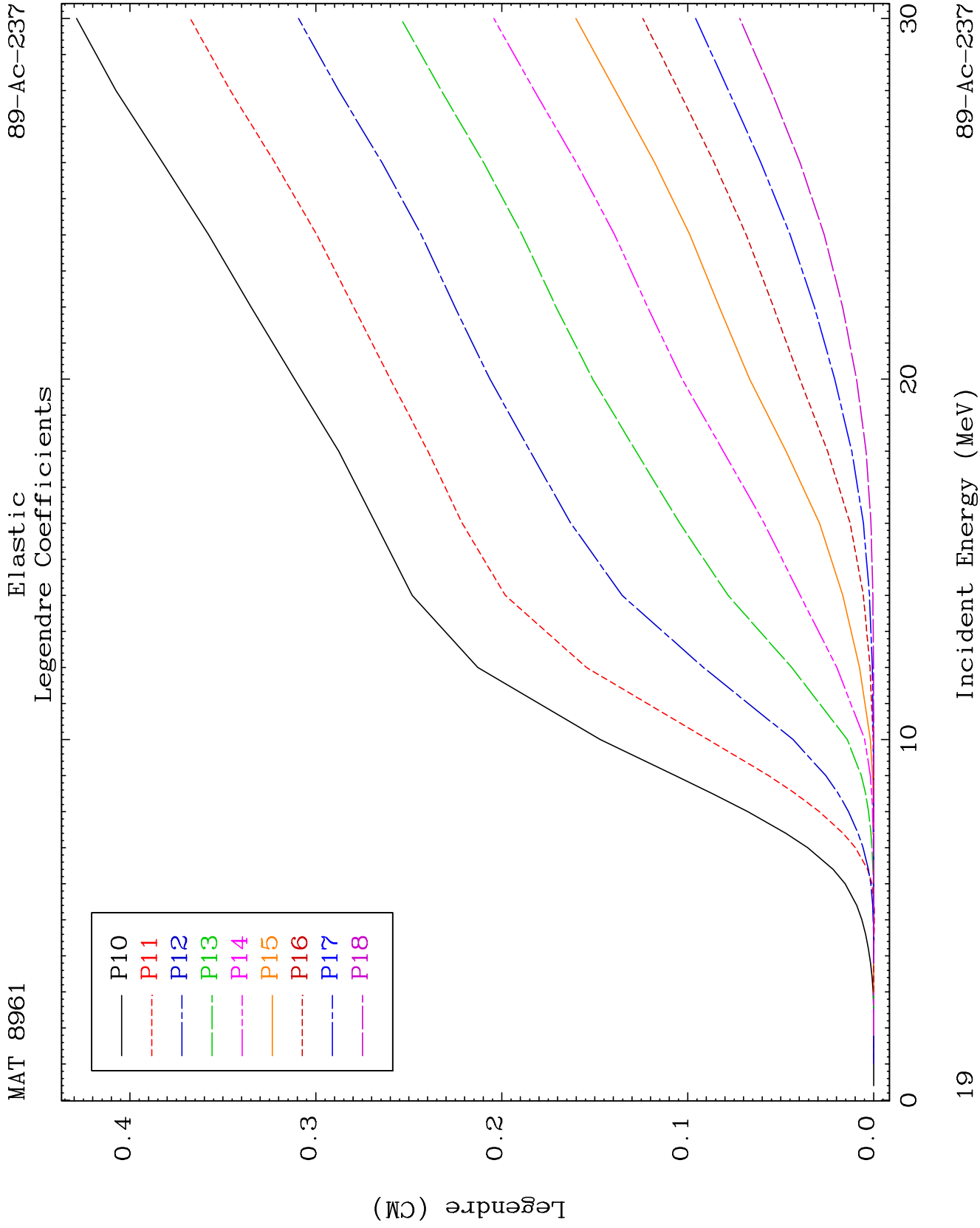


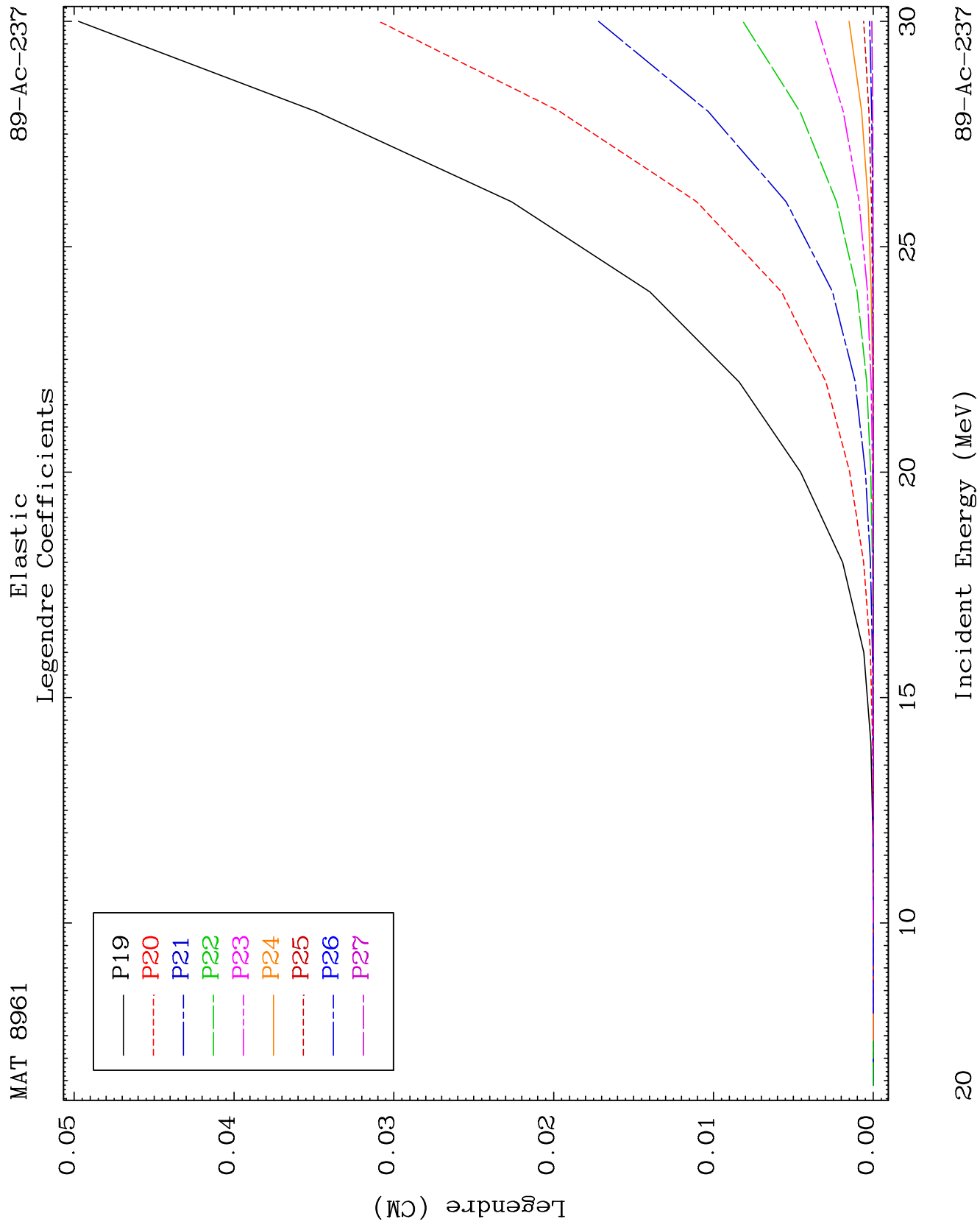
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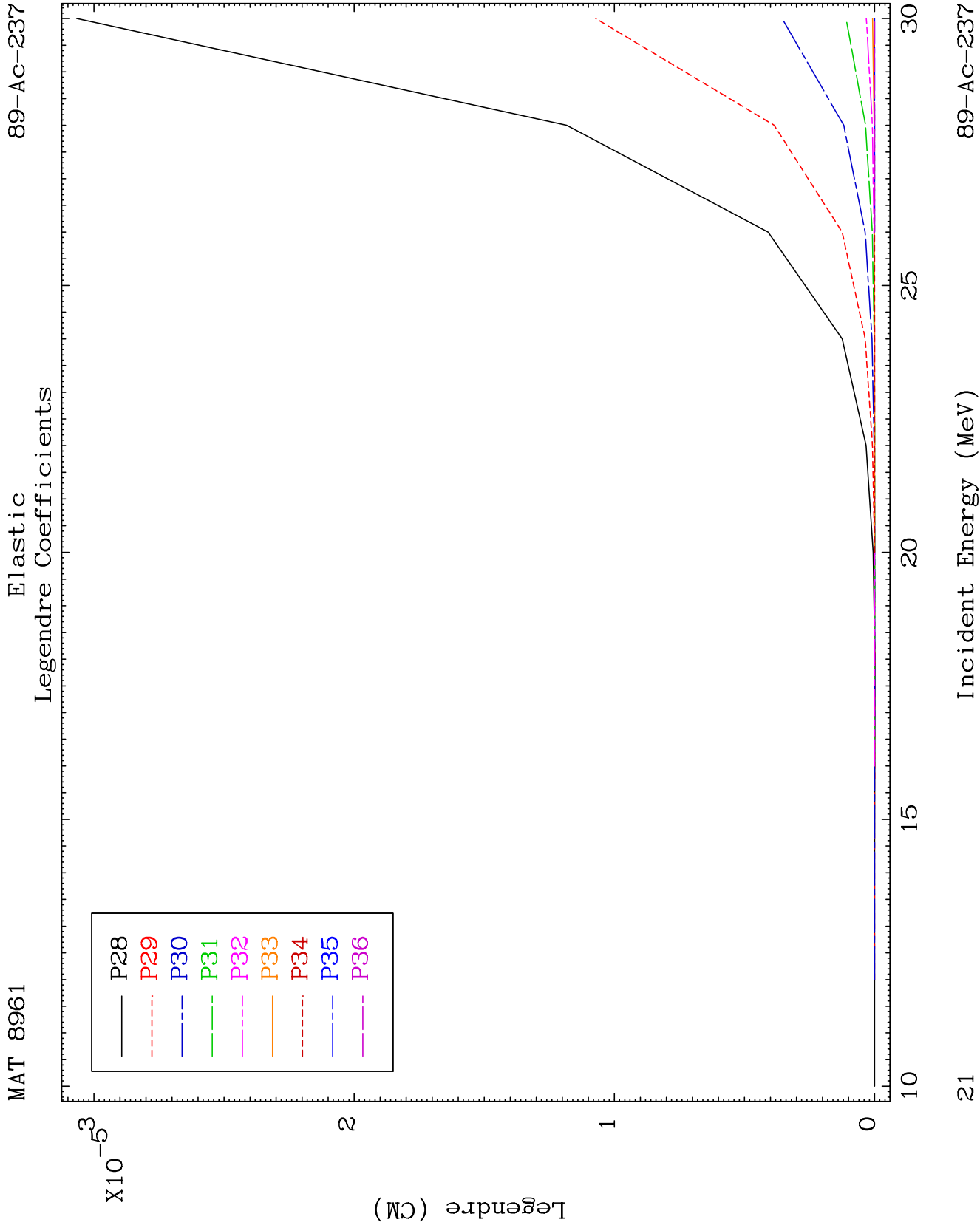
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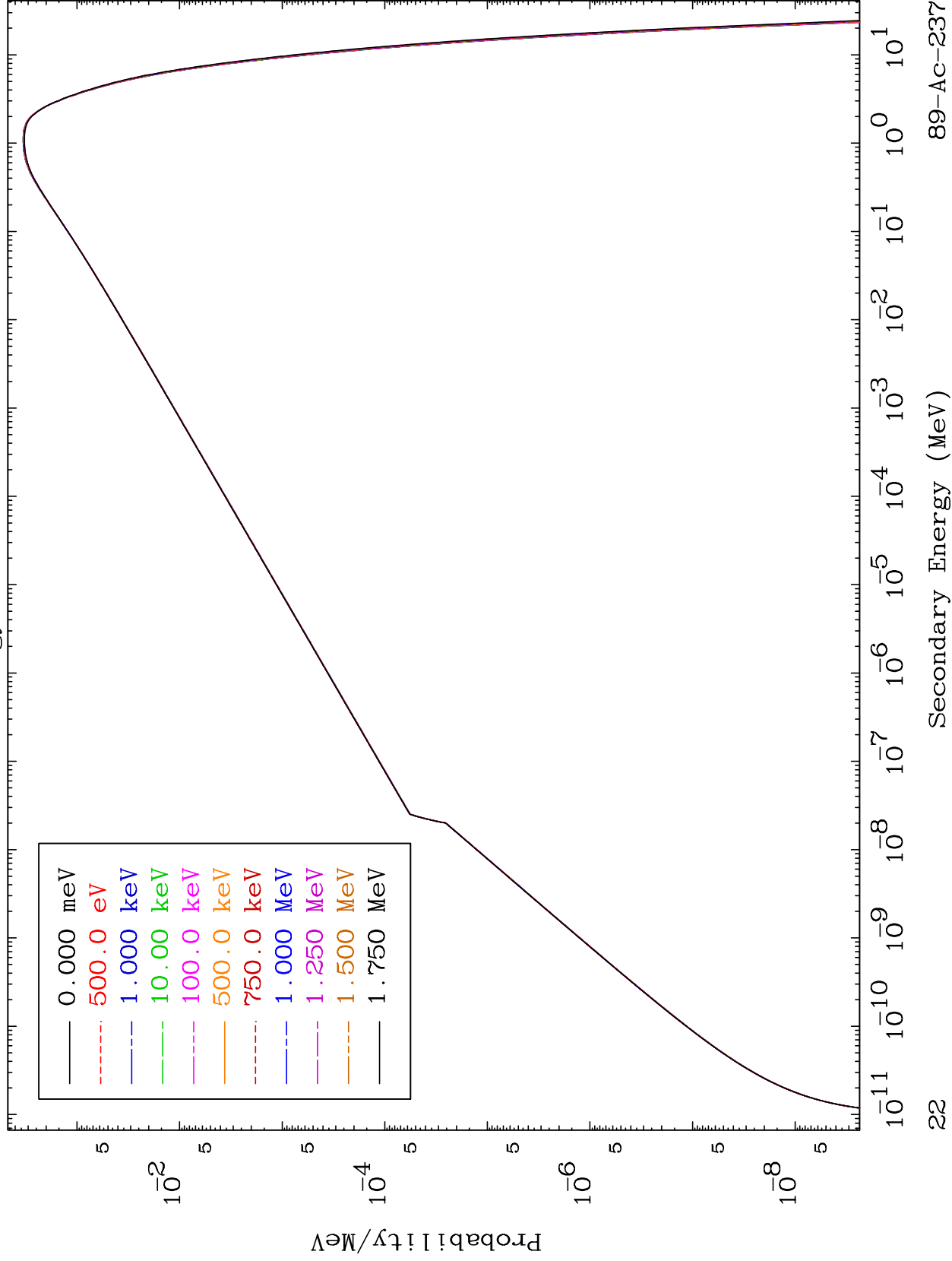




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Fission
Energy Distributions

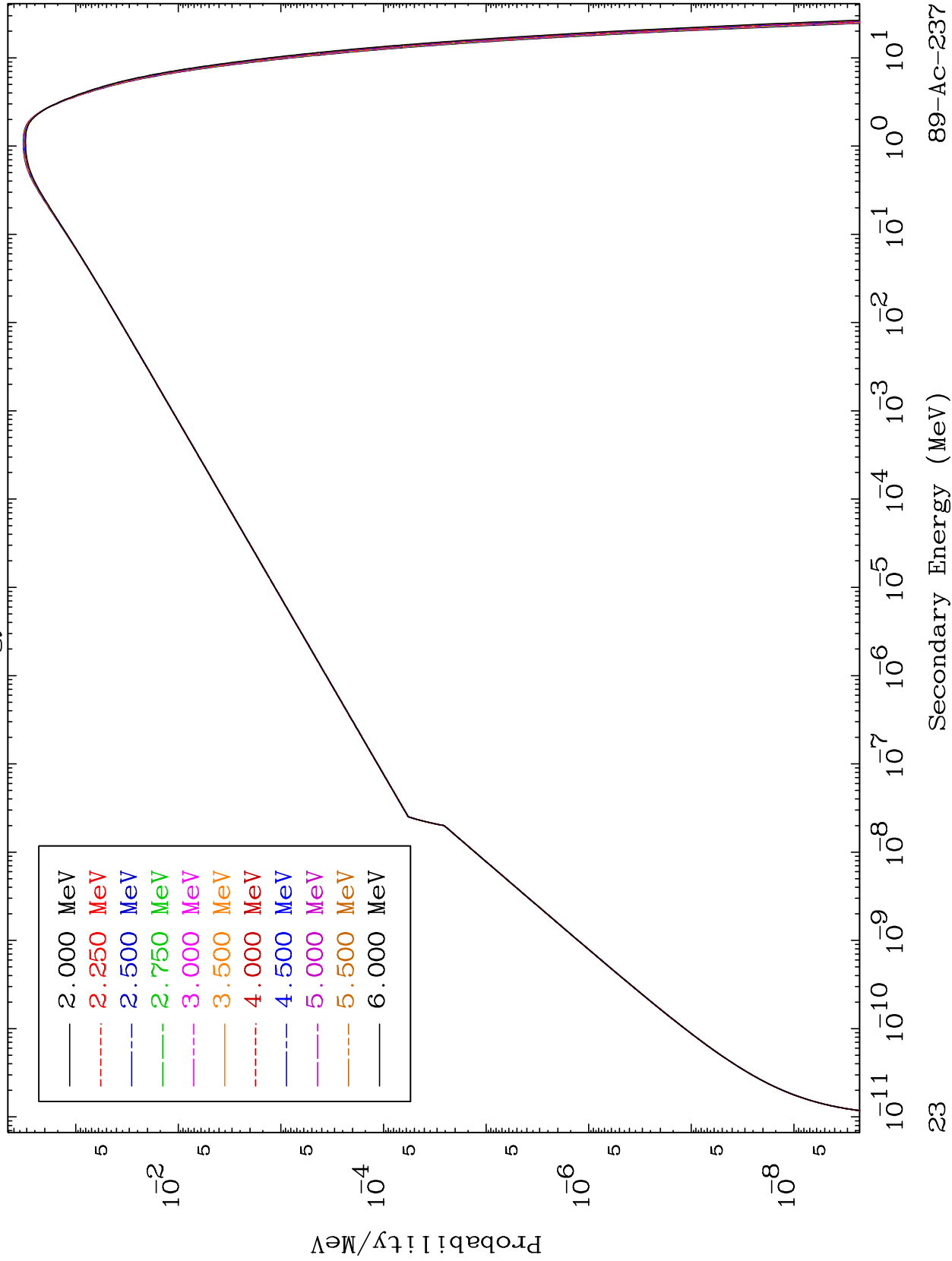
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Fission Energy Distributions

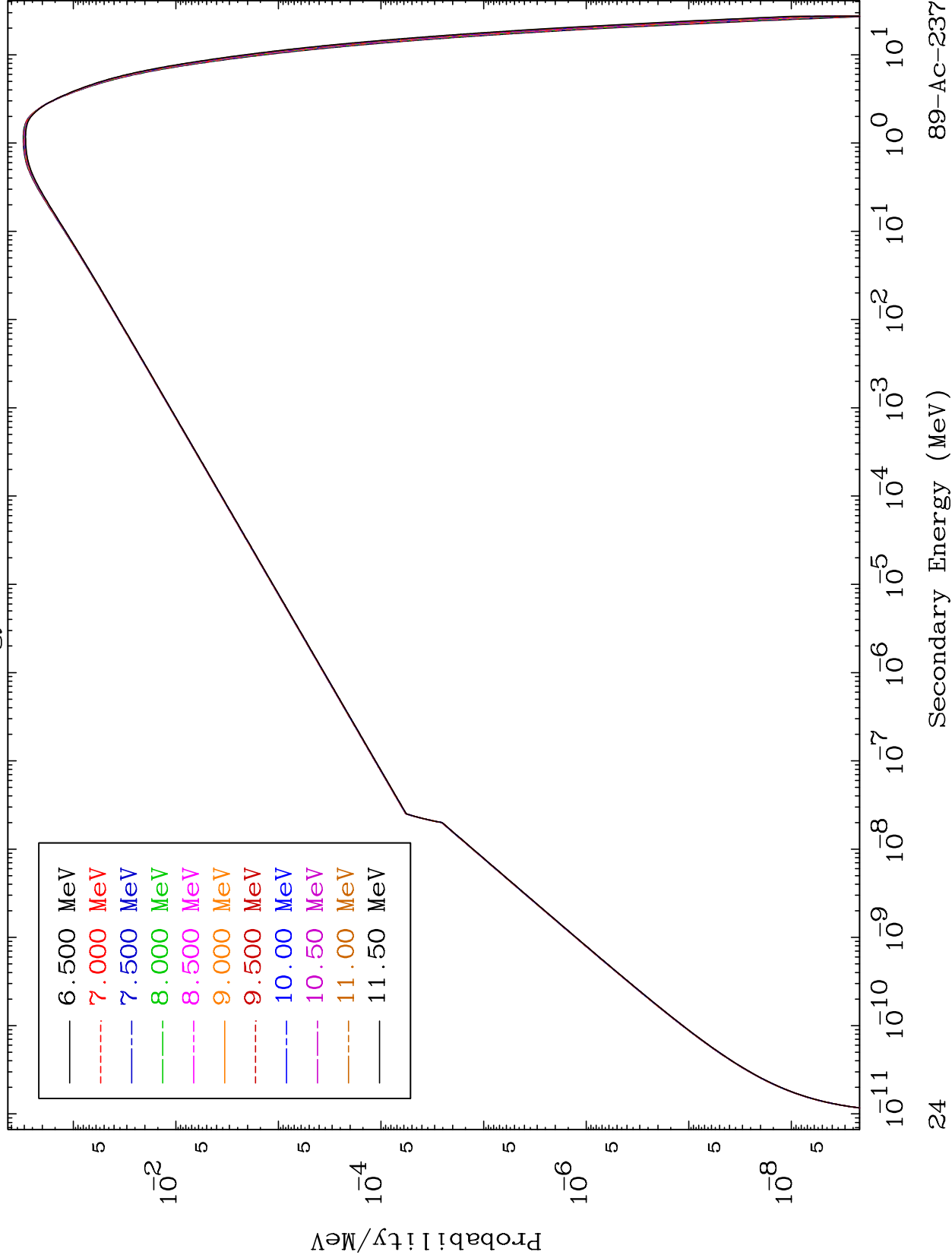
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Fission
Energy Distributions

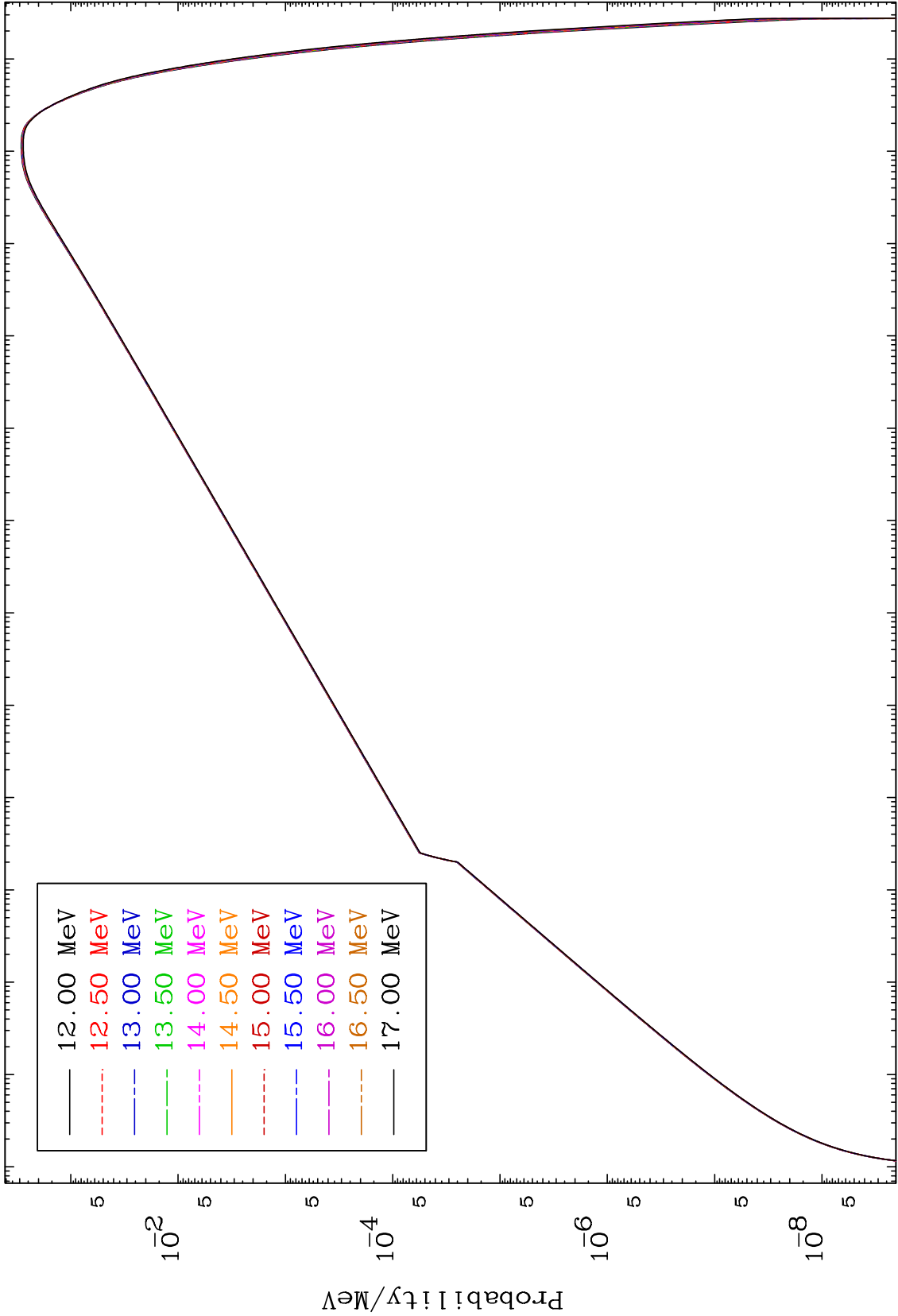
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Fission
Energy Distributions

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Secondary Energy (MeV)

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25

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Fission
Energy Distributions

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