

Program EVALPLOT
(Version 2021-1)

by

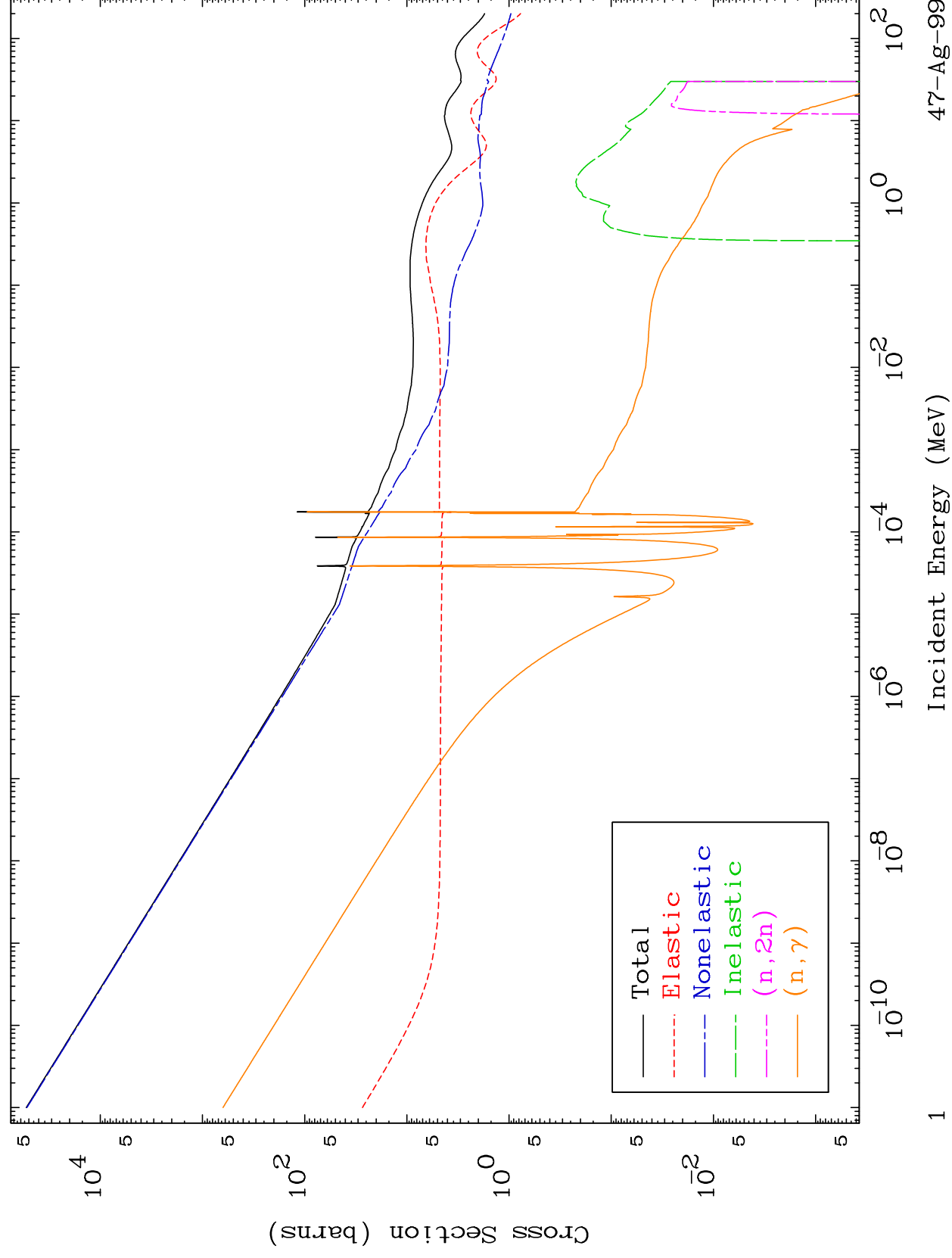
Dermott E. Cullen
(Present Contact Information)

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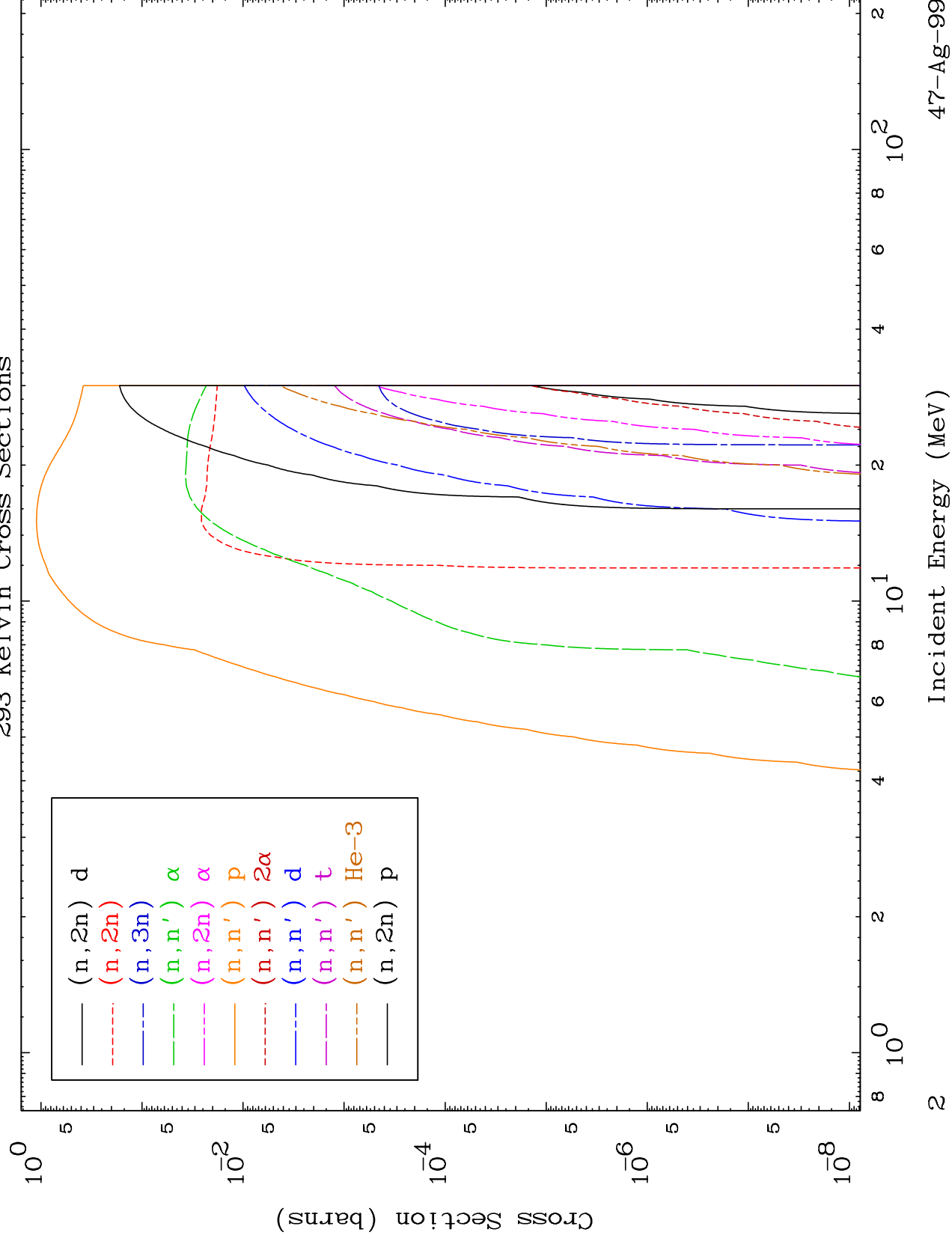
Press Mouse Button to Start

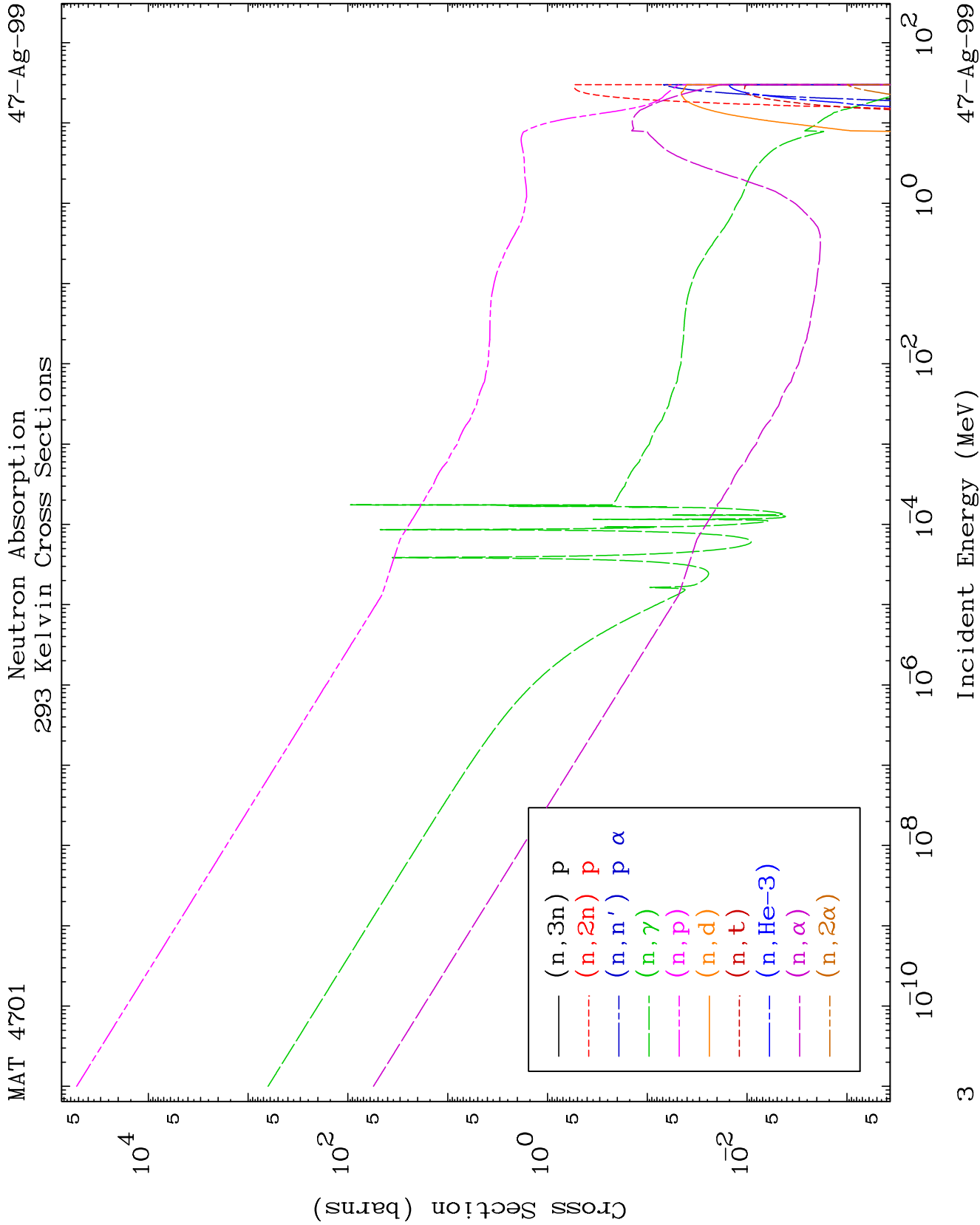


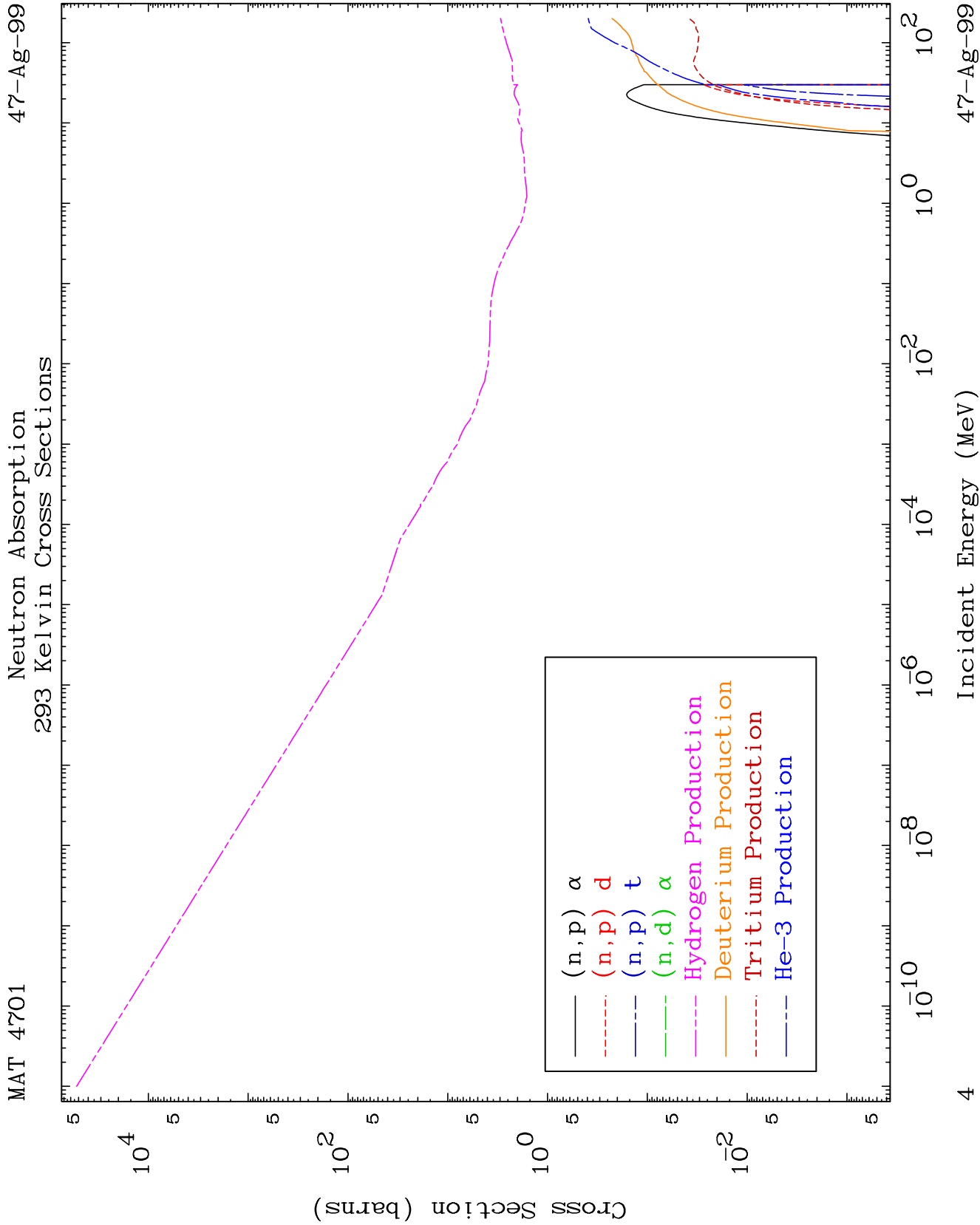
MAT 4701

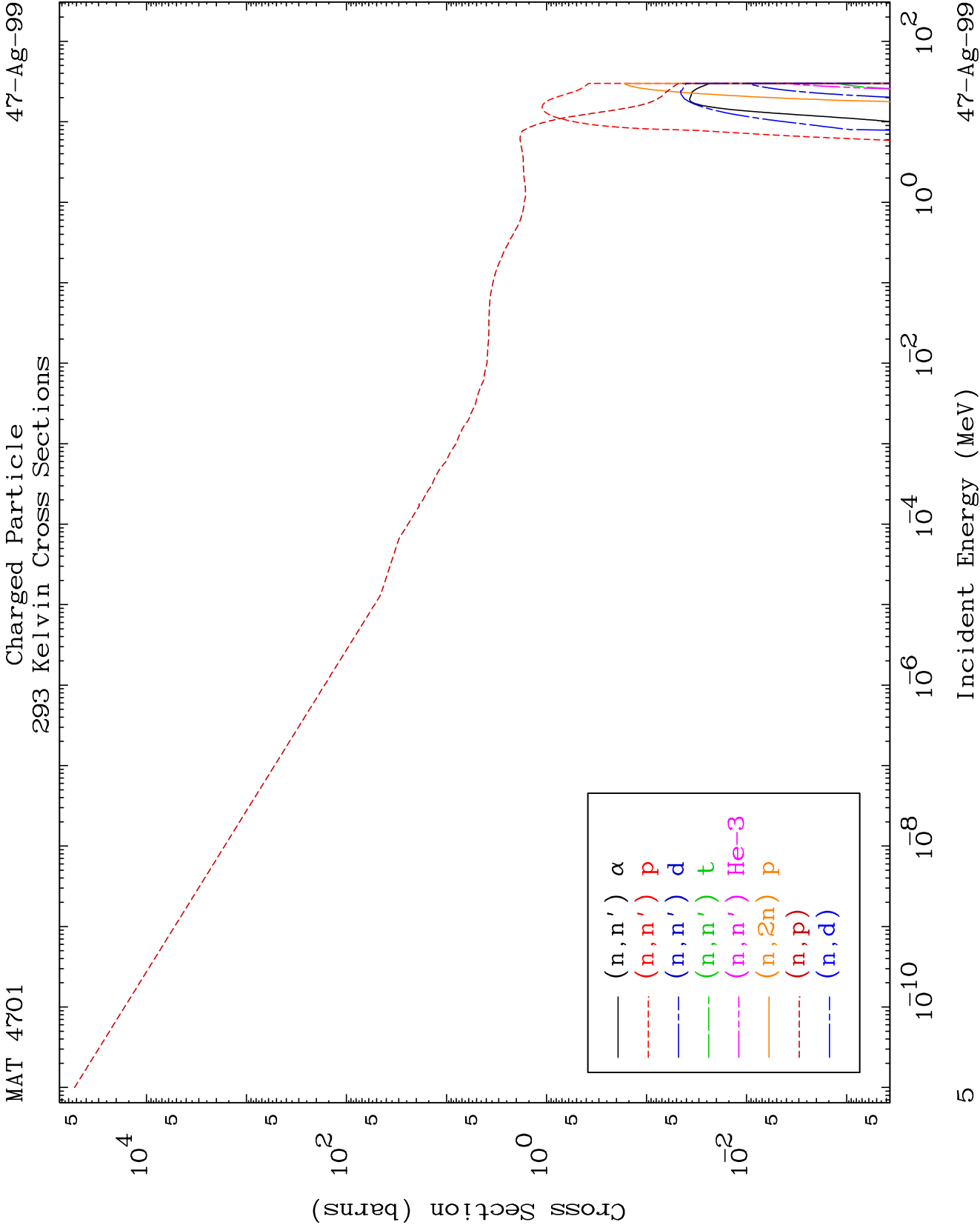
Neutron Absorption
293 Kelvin Cross Sections

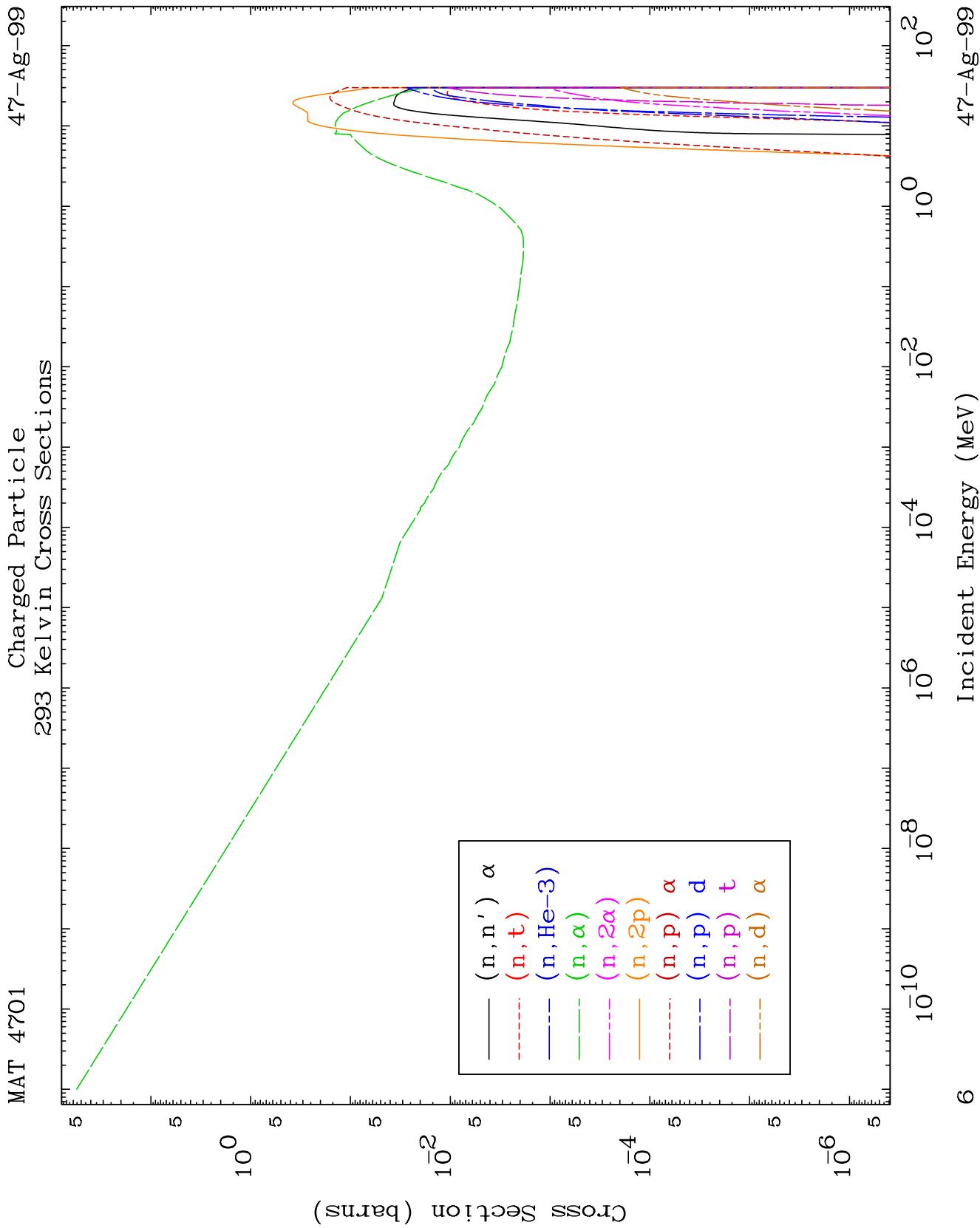
47-Ag-99







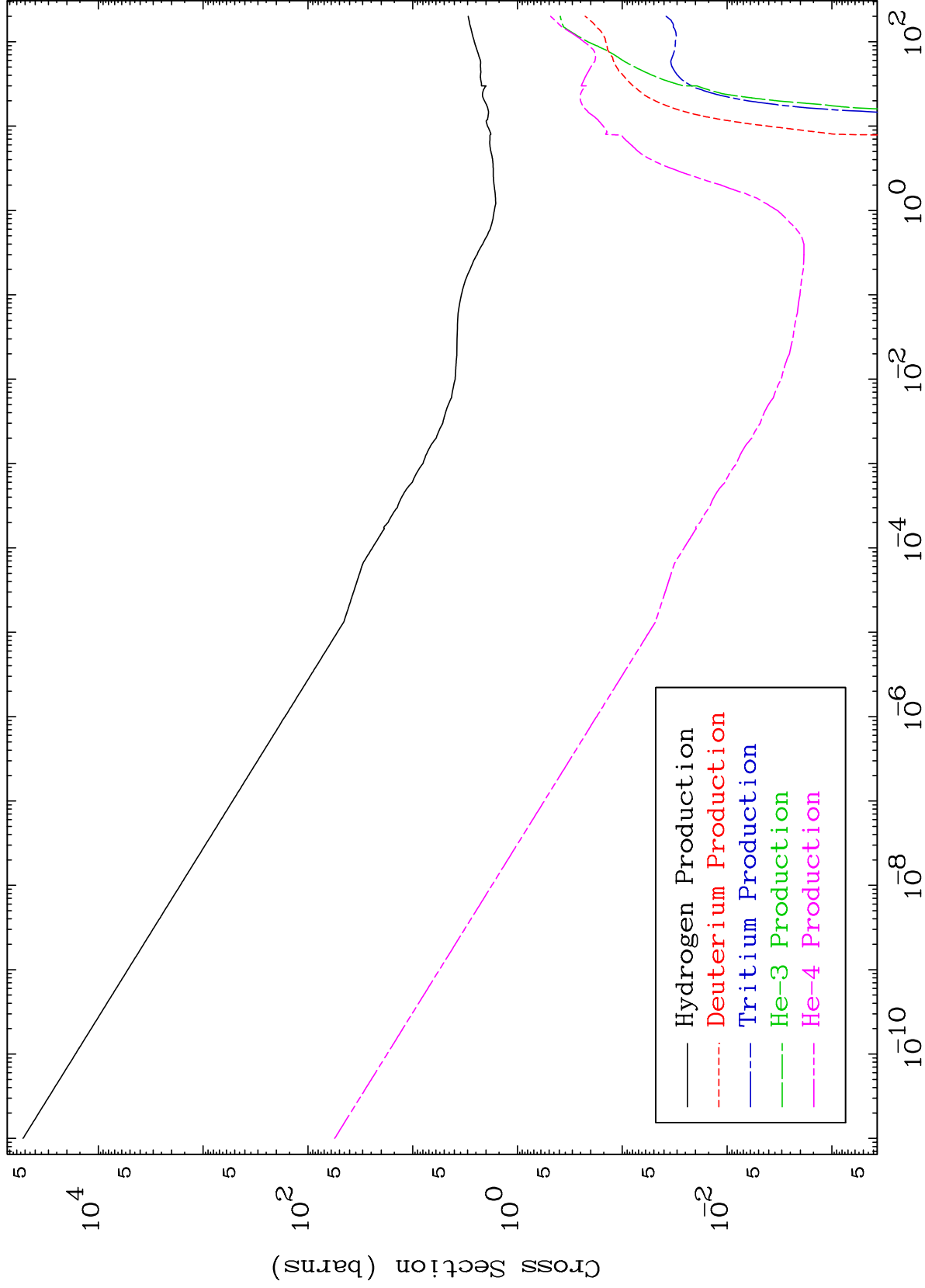


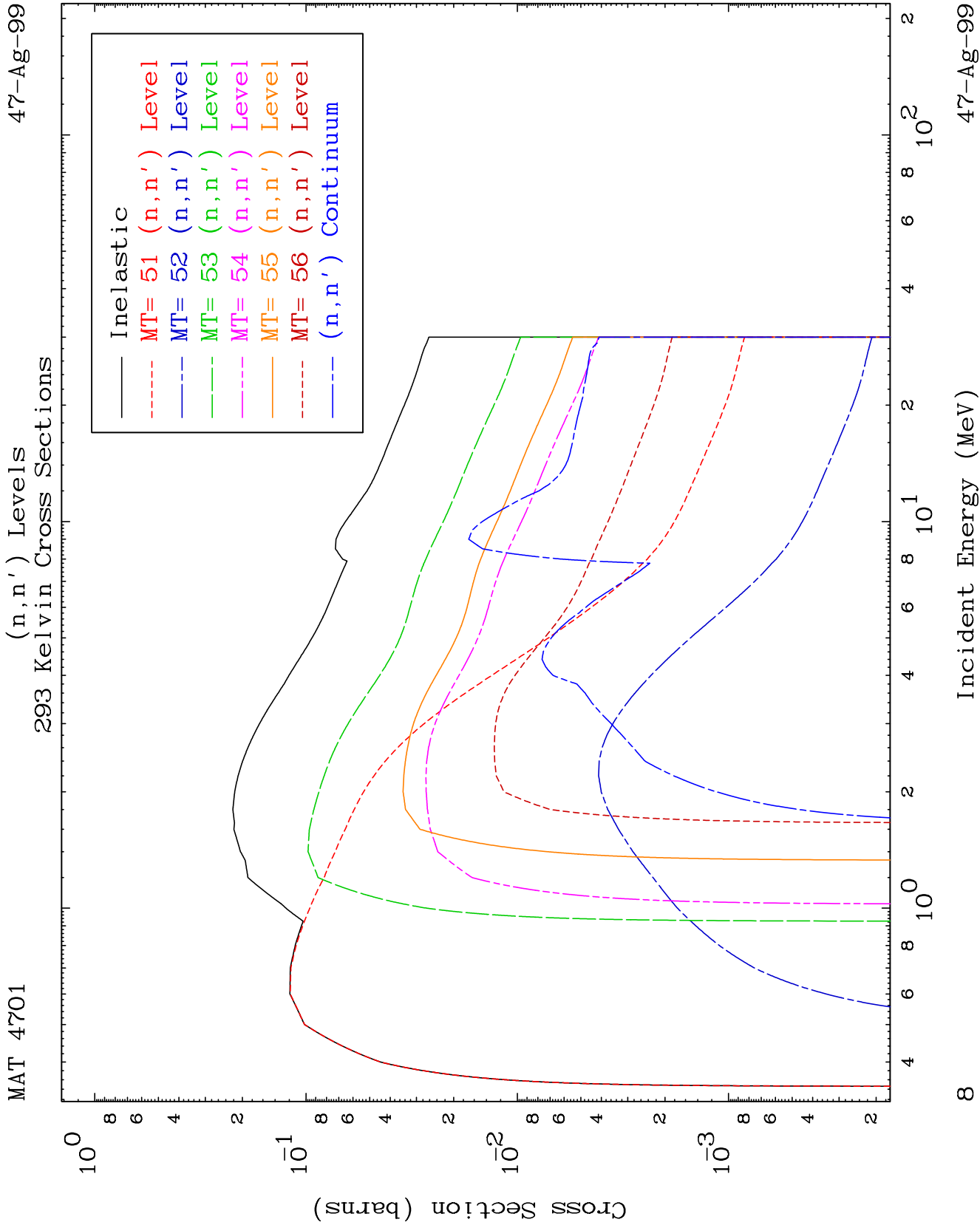


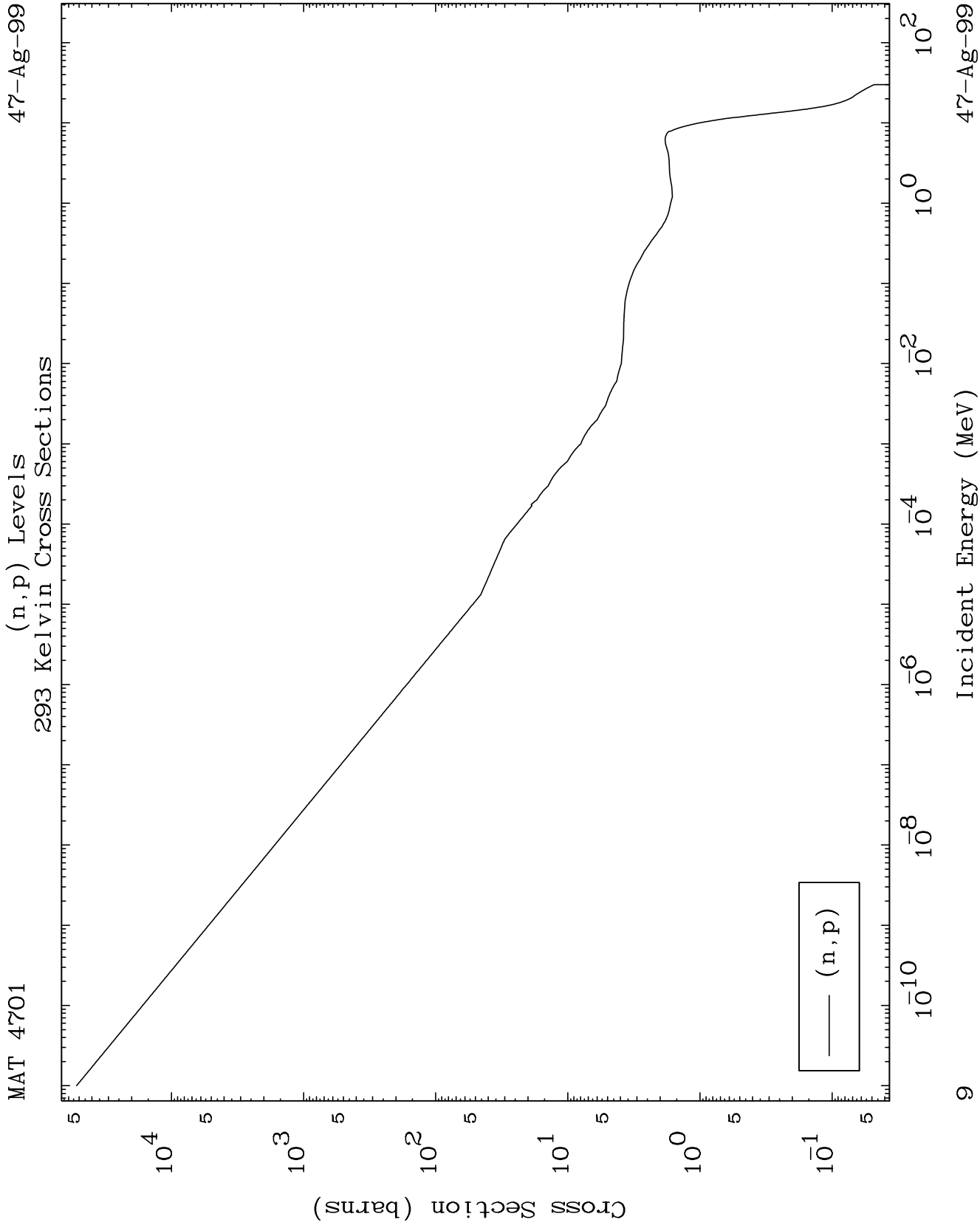
MAT 4701

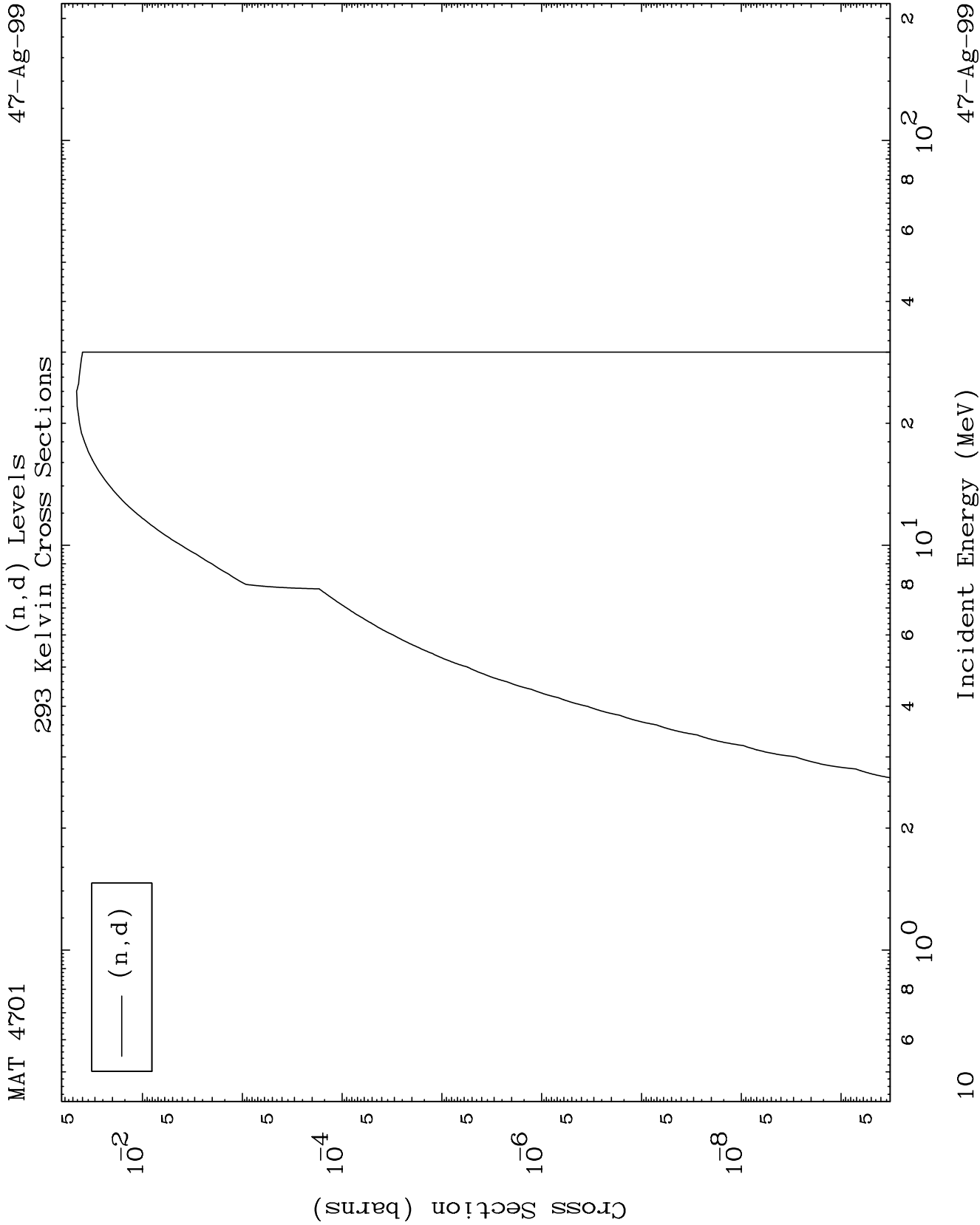
Particle Production
293 Kelvin Cross Sections

47-Ag-99





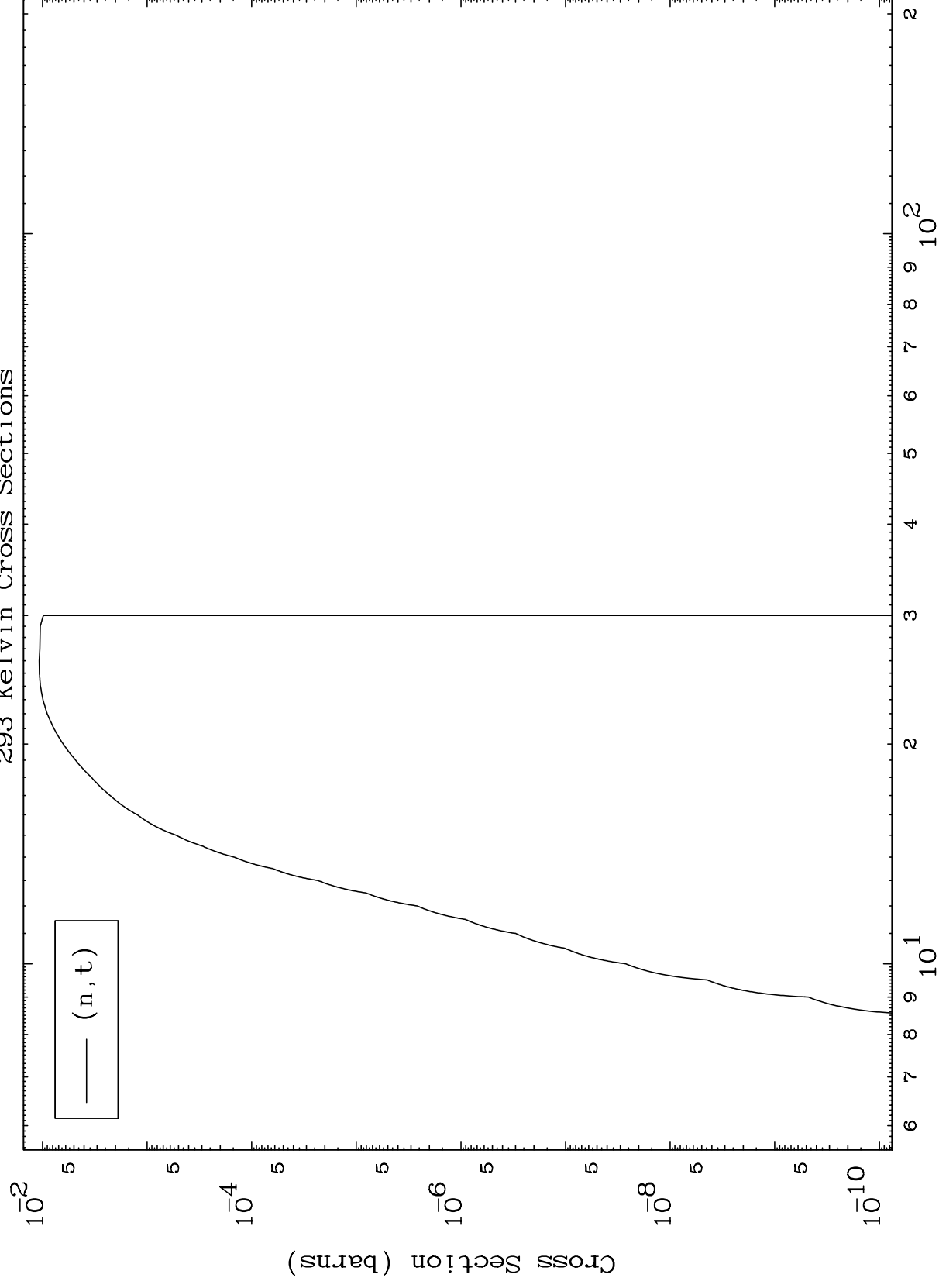




MAT 4701

47-Ag-99

(n, t) Levels
293 Kelvin Cross Sections



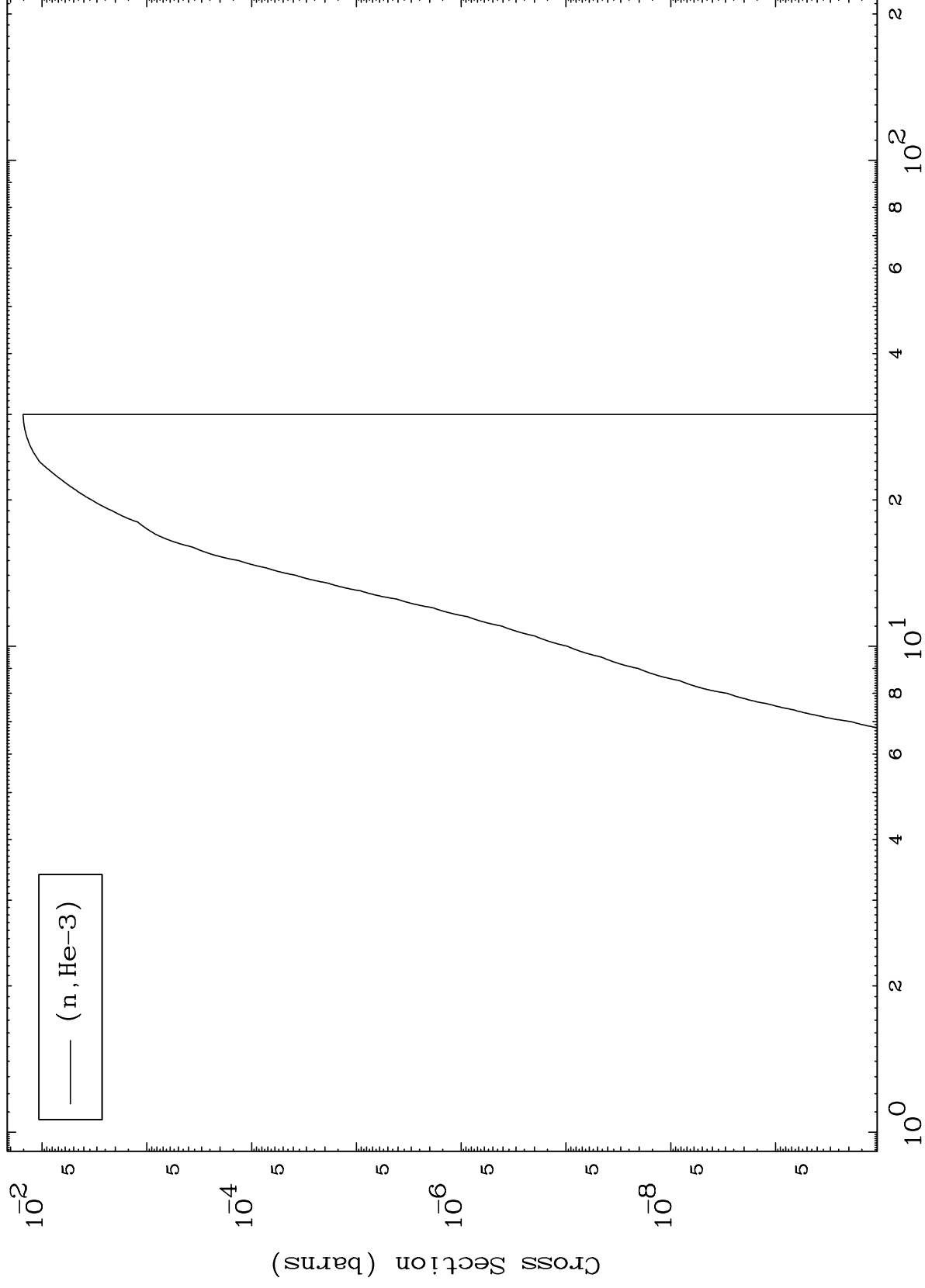
11

47-A8-99

MAT 4701

(n,He3) Levels
293 Kelvin Cross Sections

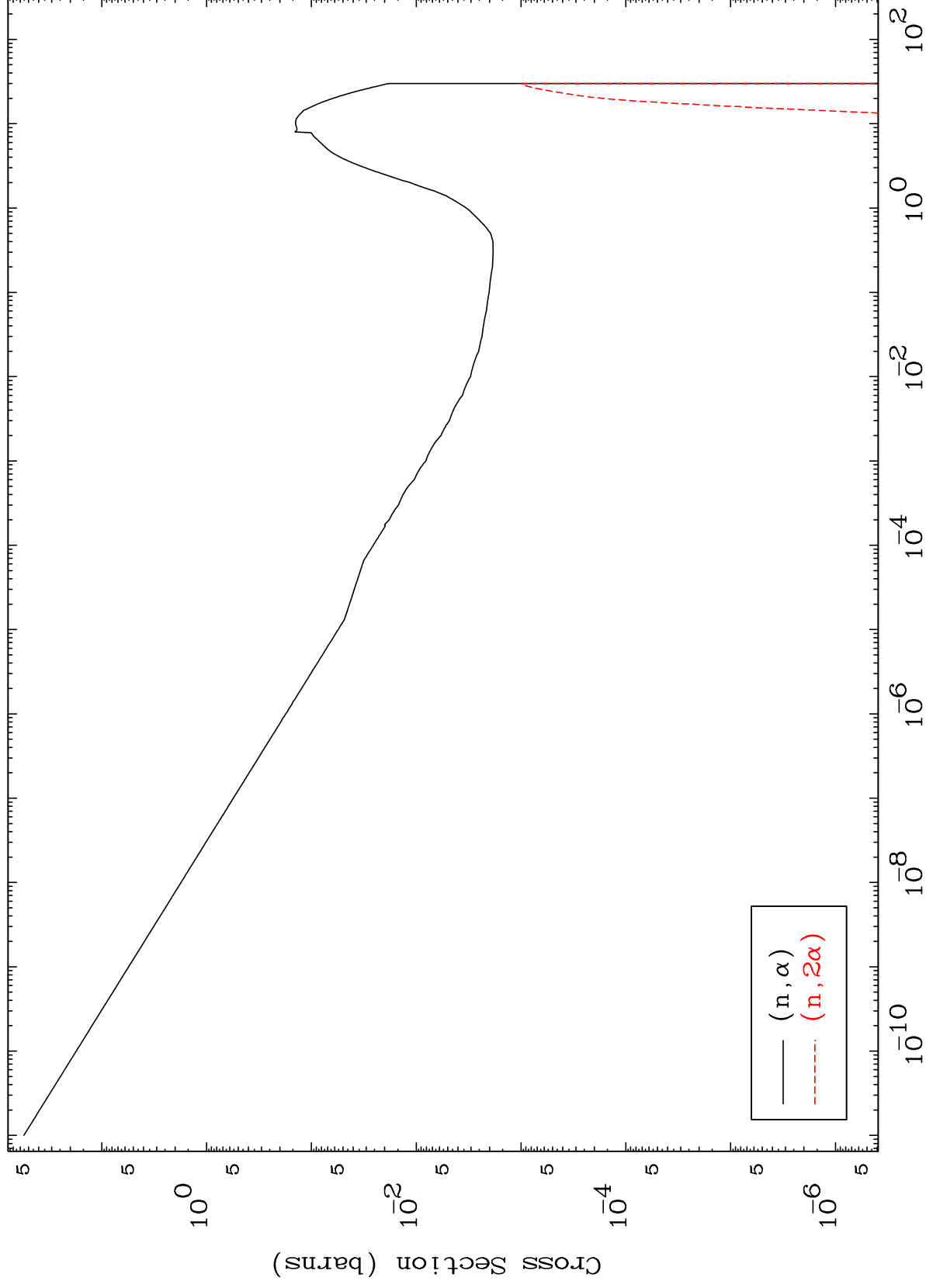
47-Ag-99



MAT 4701

(n, α) Levels
293 Kelvin Cross Sections

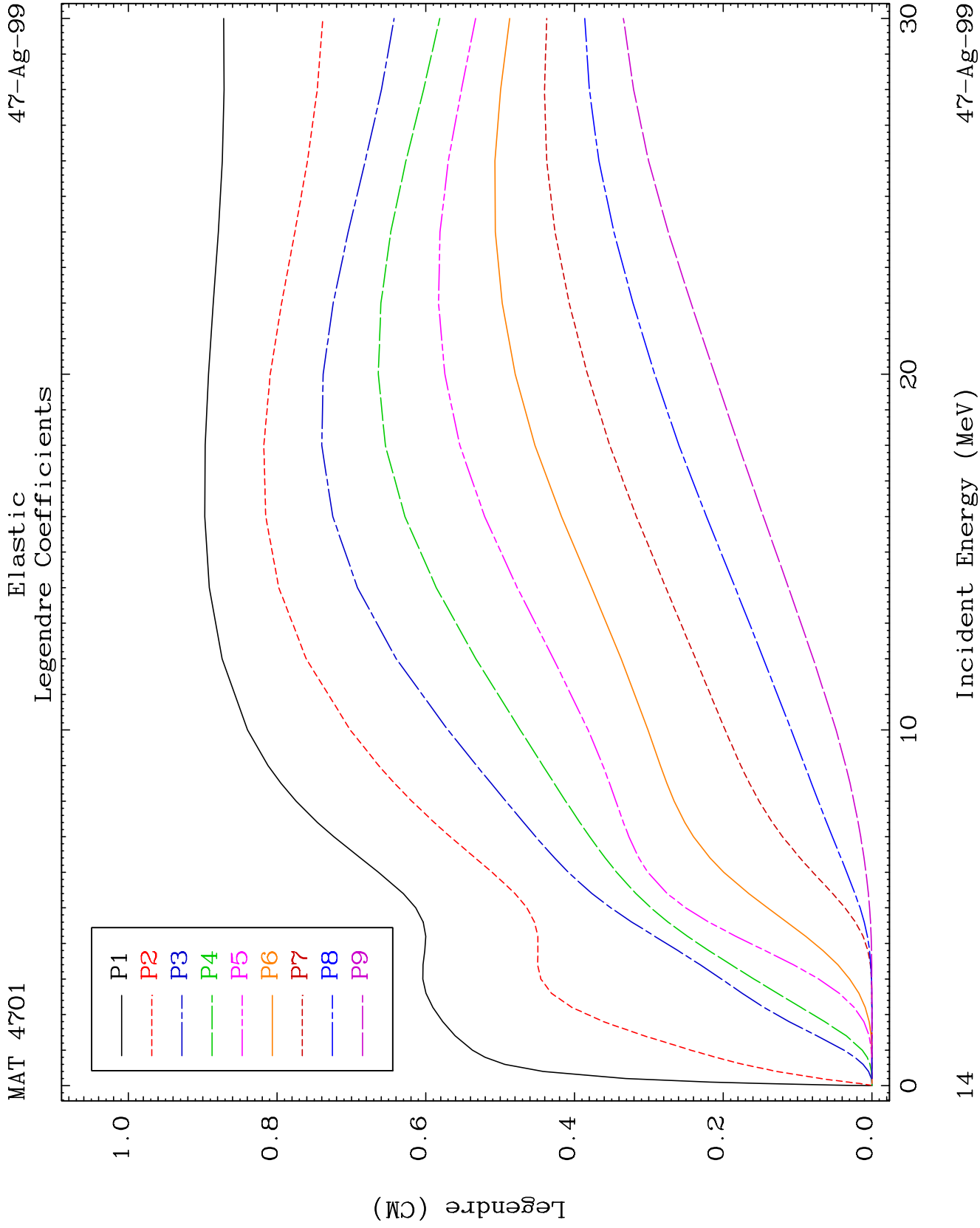
47-Ag-99

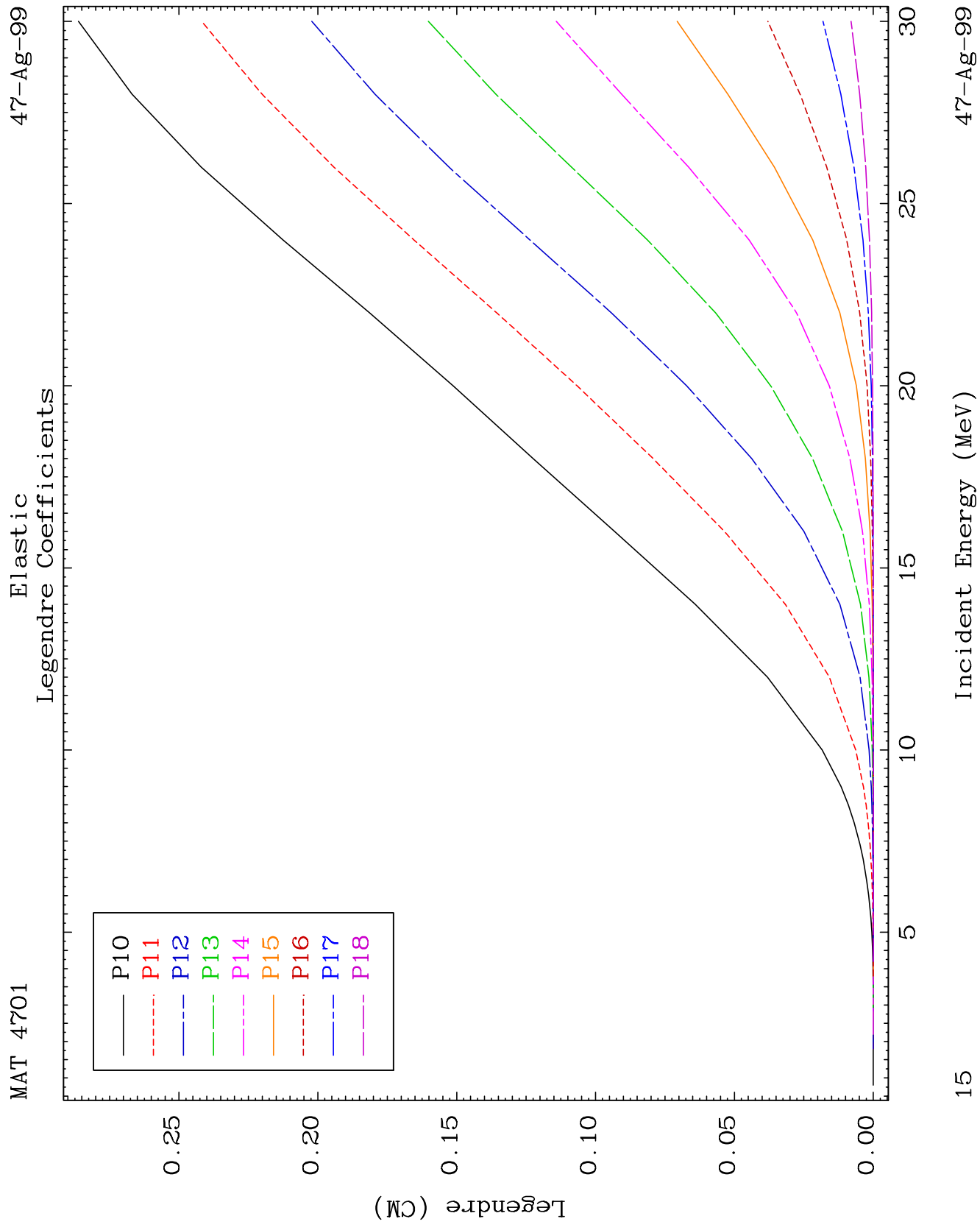


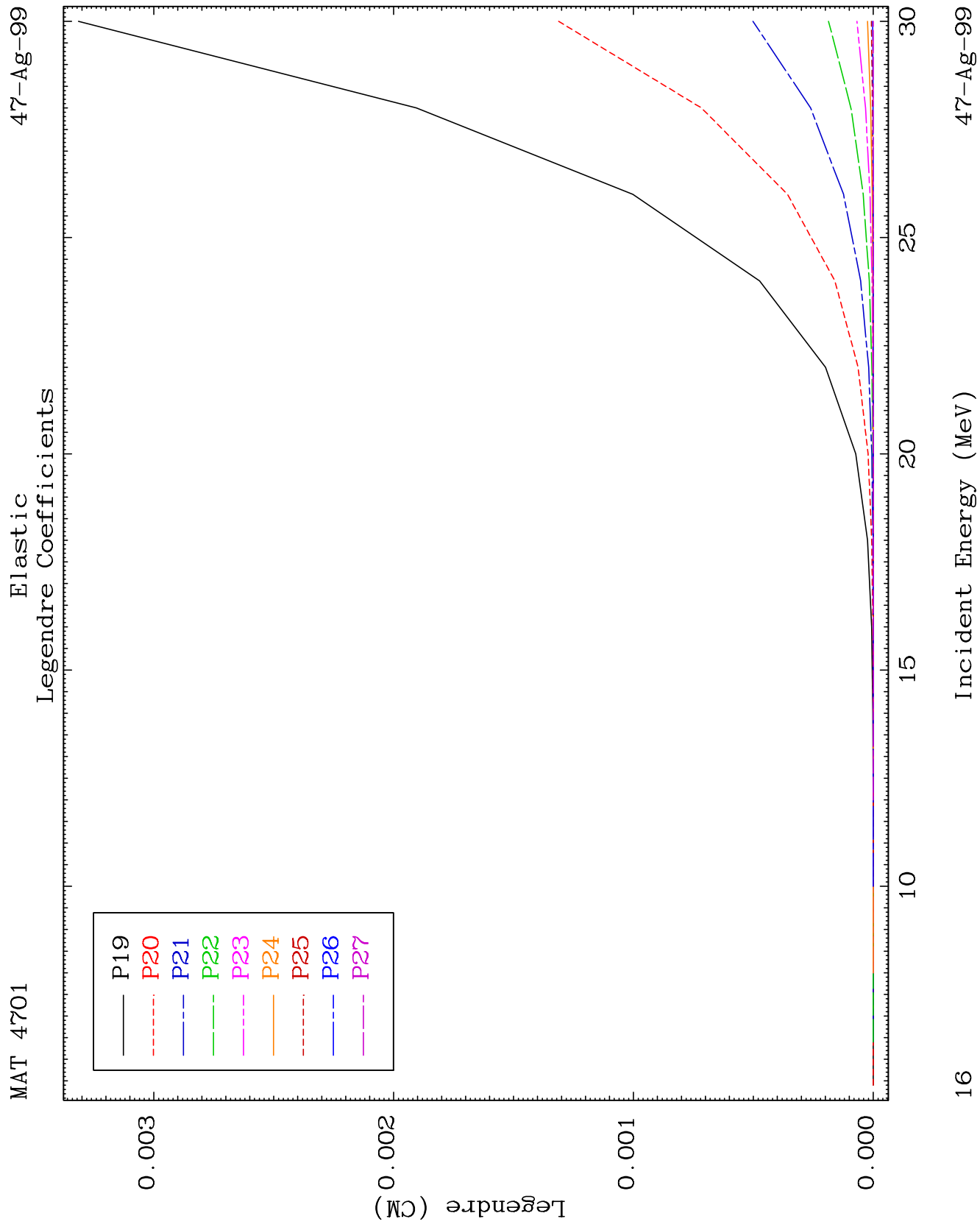
13

Incident Energy (MeV)

47-Ag-99



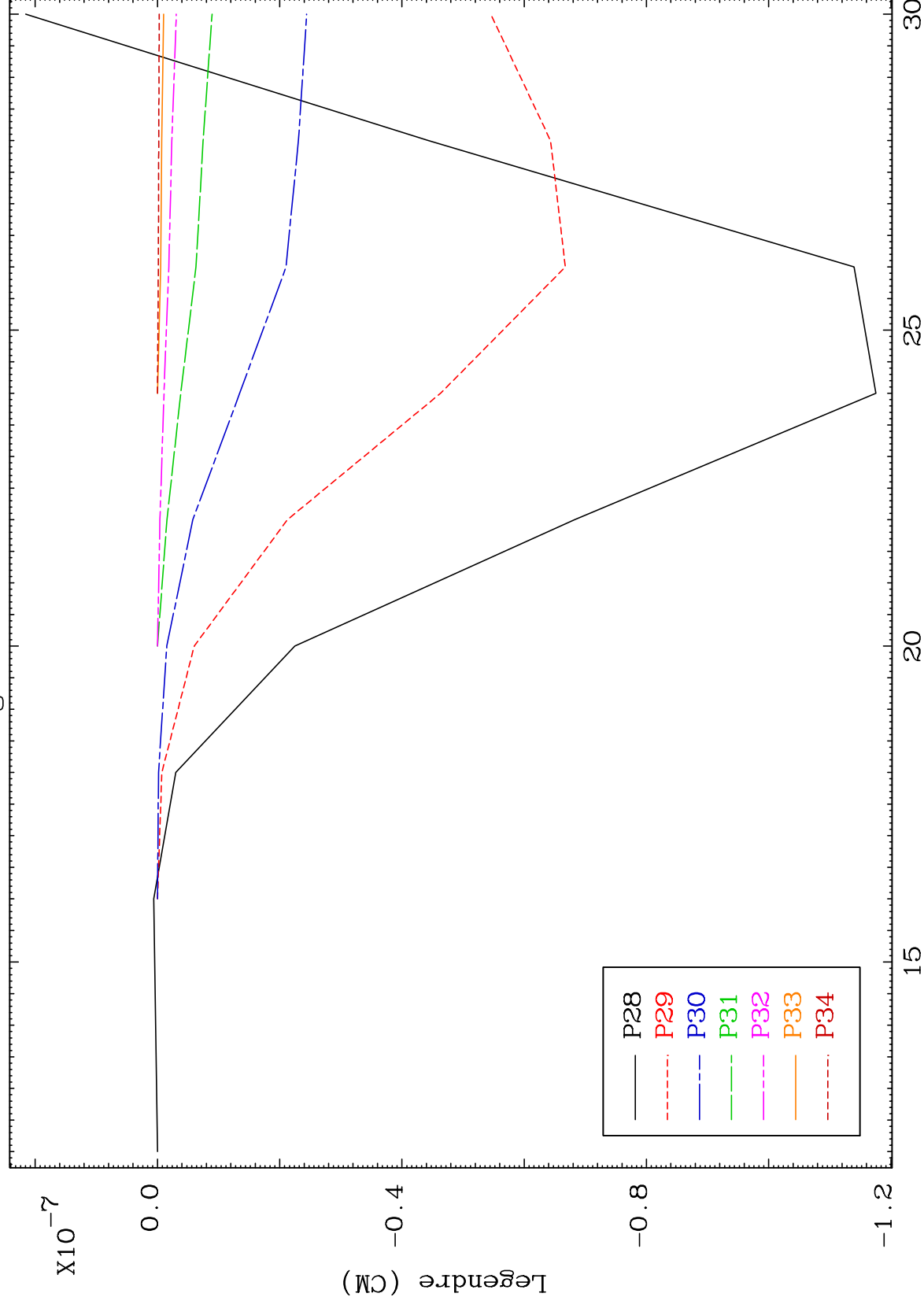




MAT 4701

47-Ag-99

Elastic Legendre Coefficients



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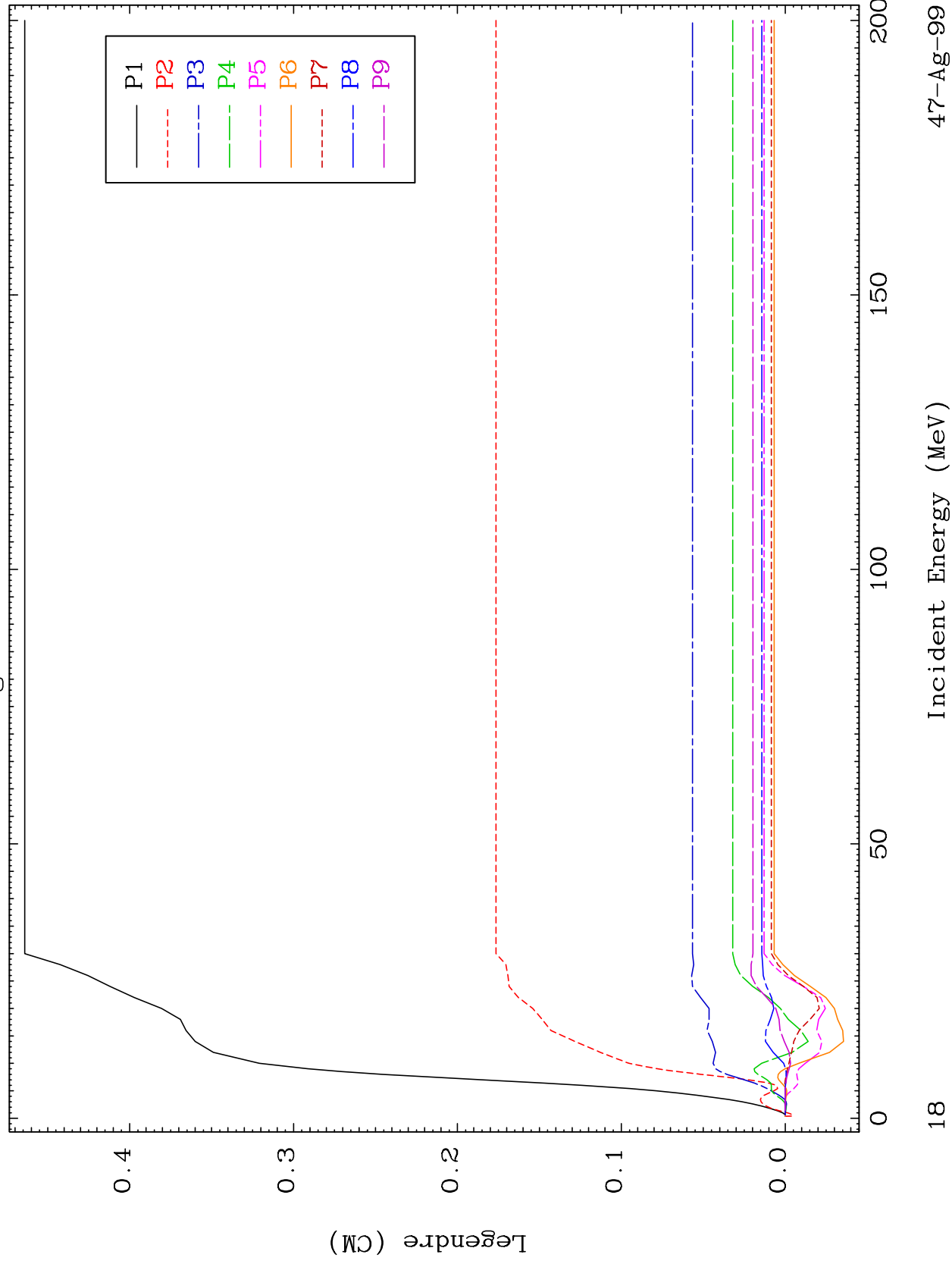
Incident Energy (MeV)

47-A8-99

MAT 4701

MT= 51 (n, n') Level
Legendre Coefficients

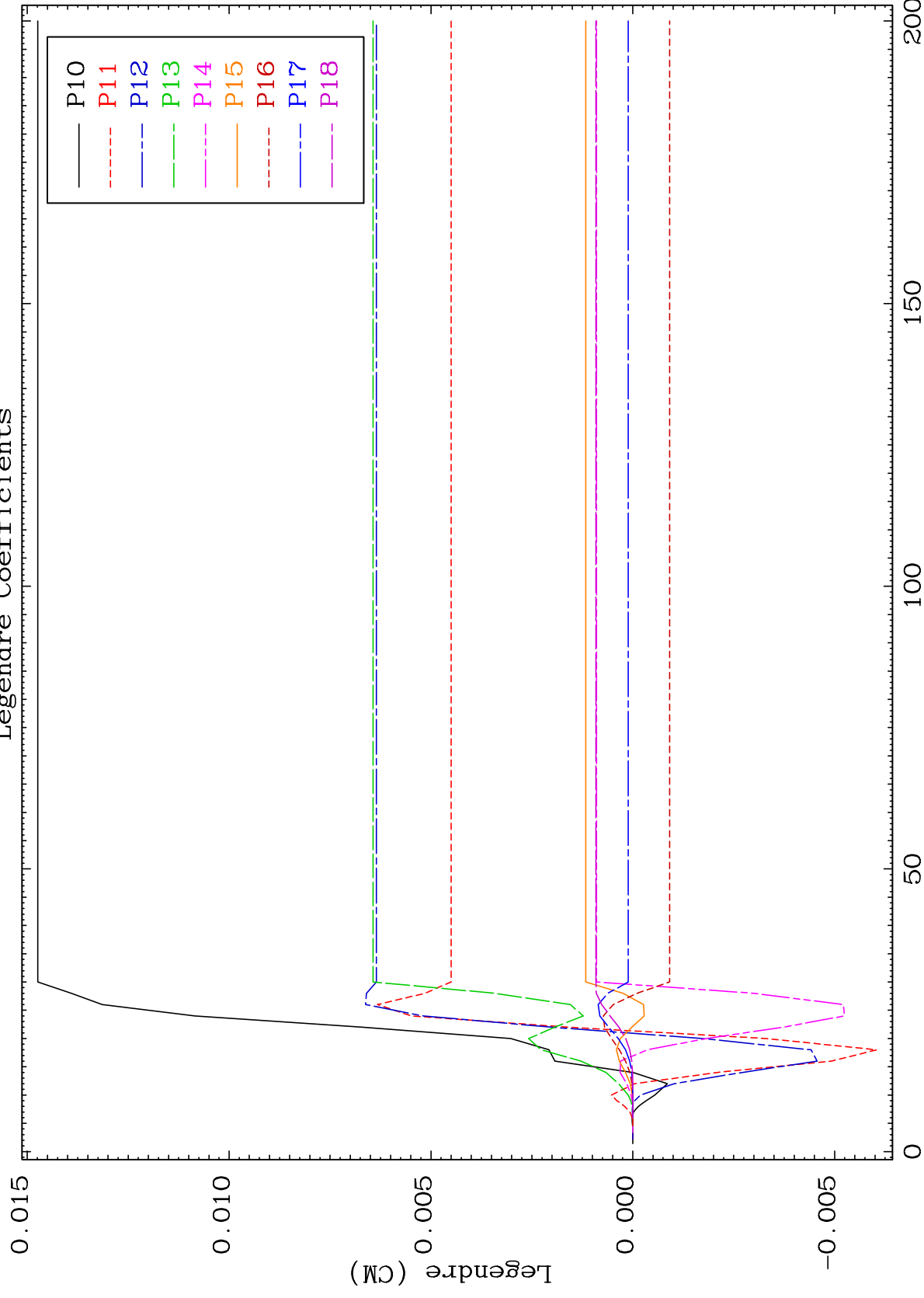
47-Ag-99



MAT 4701

MT= 51 (n, n') Level
Legendre Coefficients

47-Ag-99



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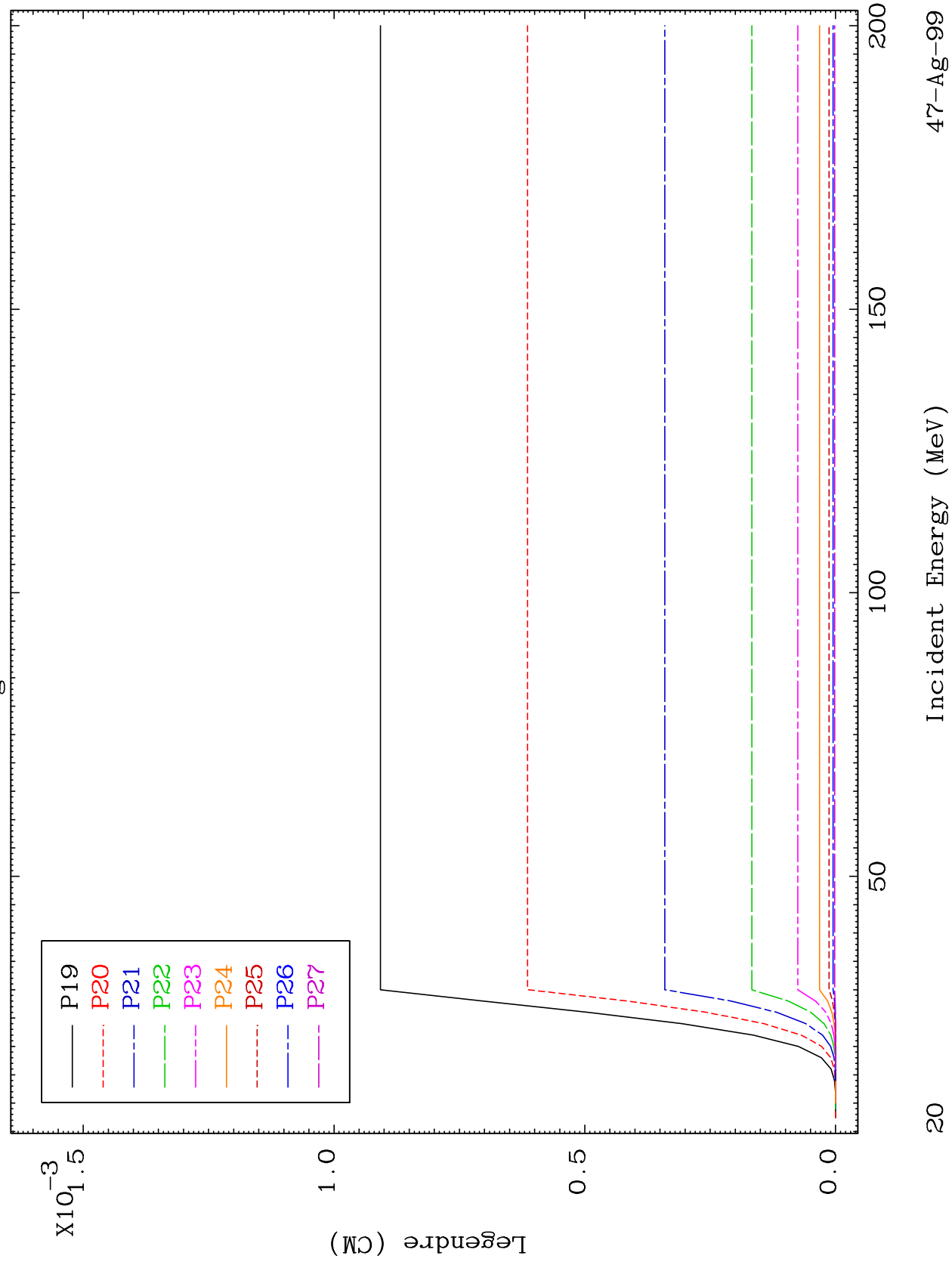
Incident Energy (MeV)

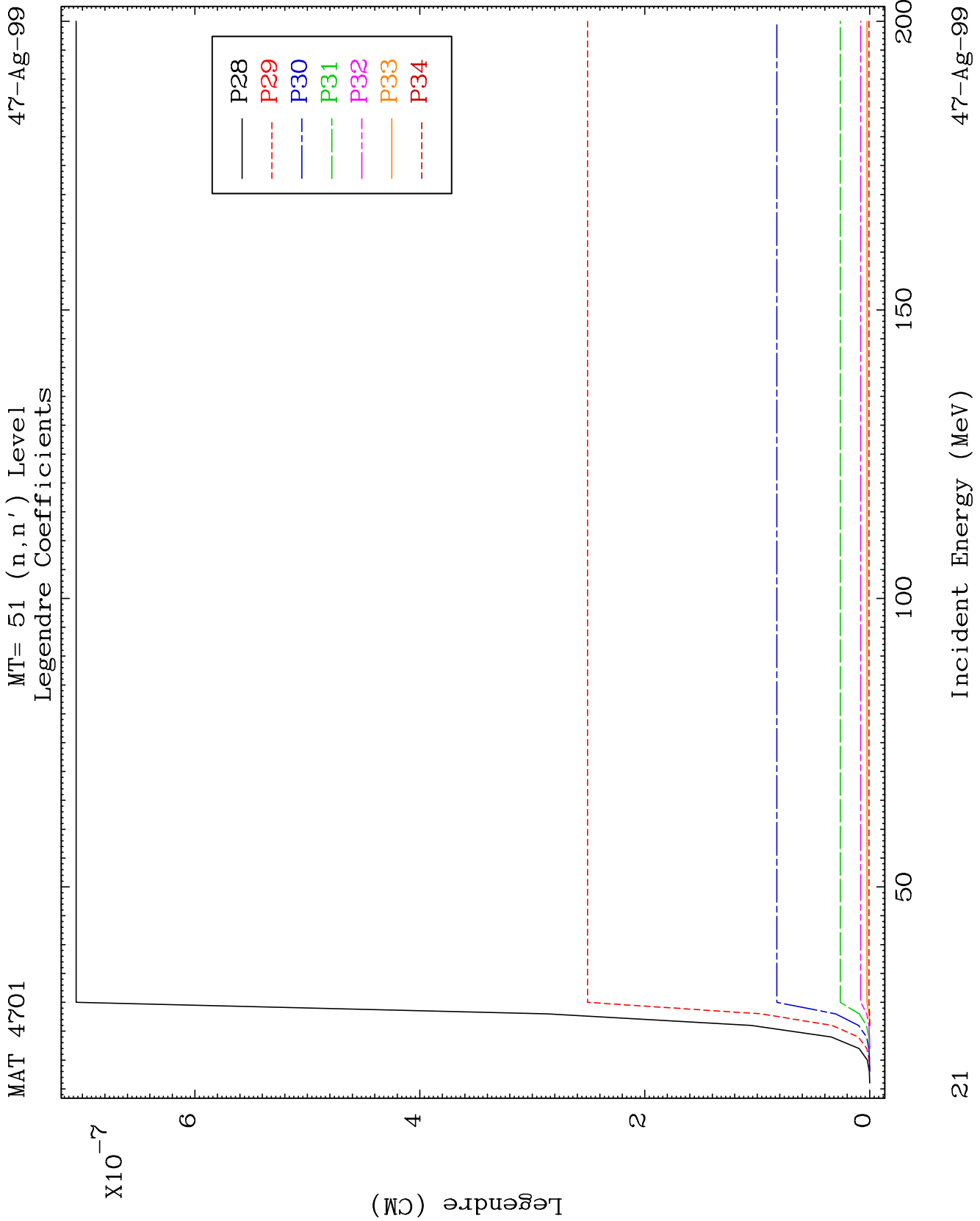
47-A8-99

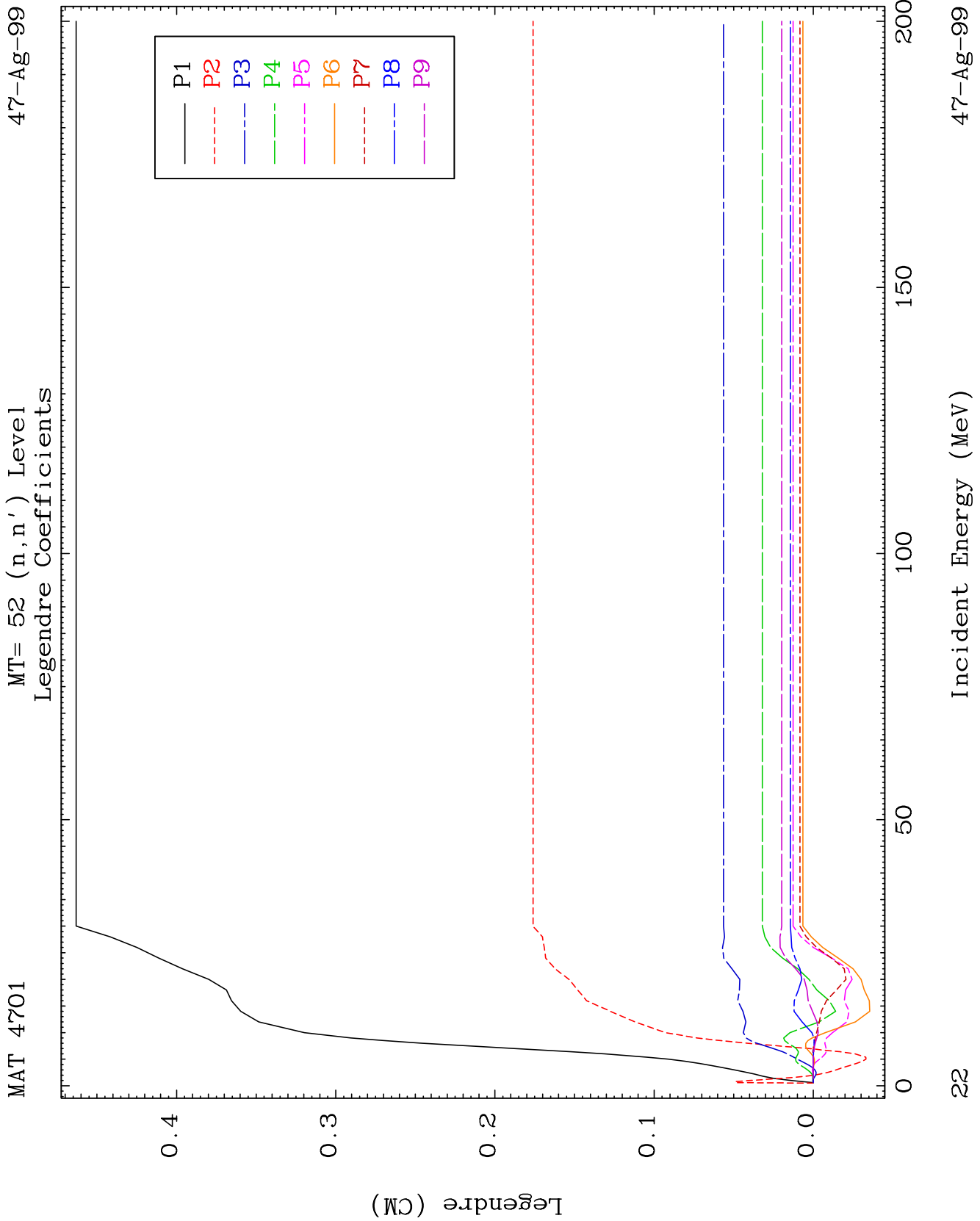
MAT 4701

MT= 51 (n, n') Level
Legendre Coefficients

47-Ag-99



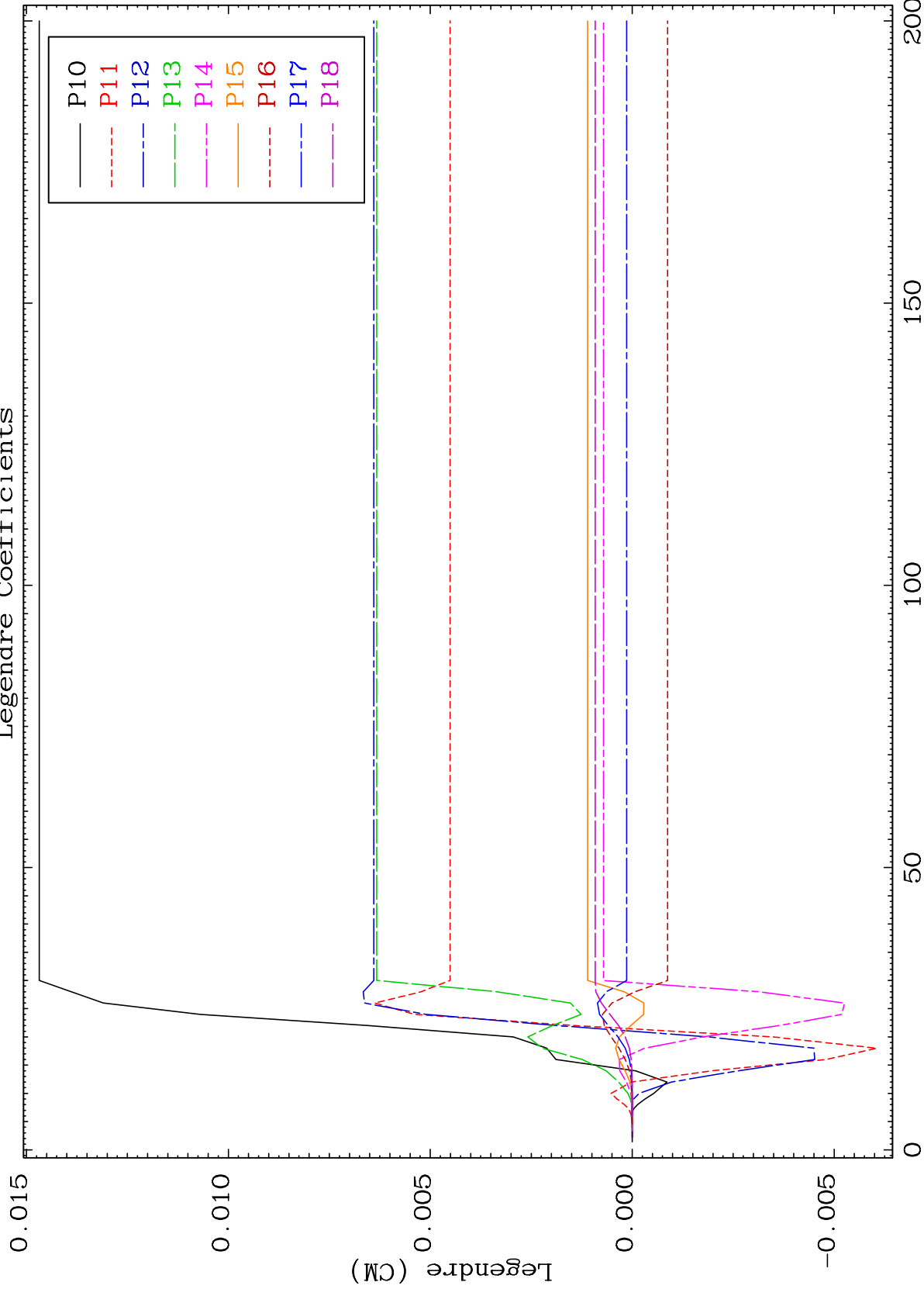




MAT 4701

MT= 52 (n, n') Level
Legendre Coefficients

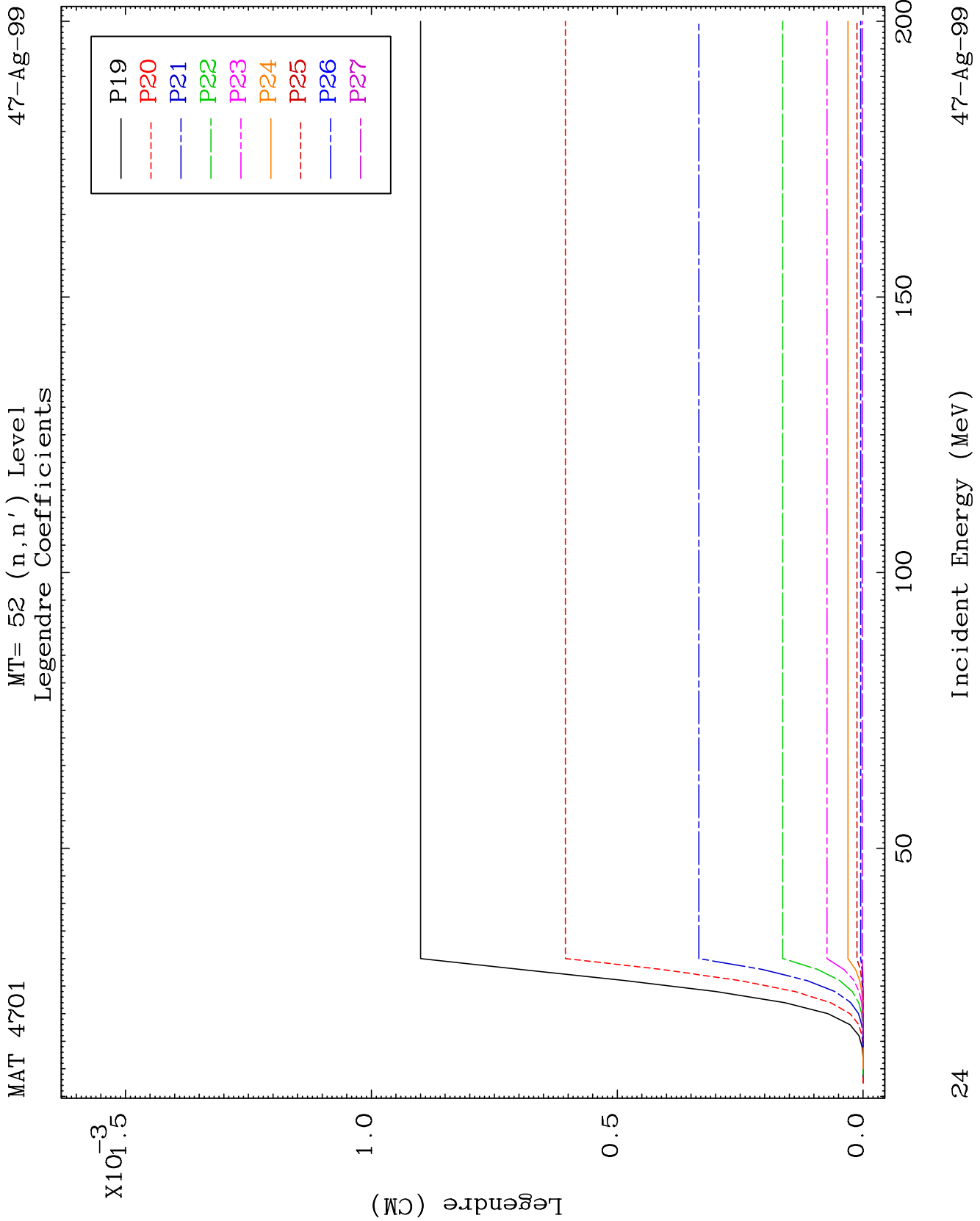
47-Ag-99

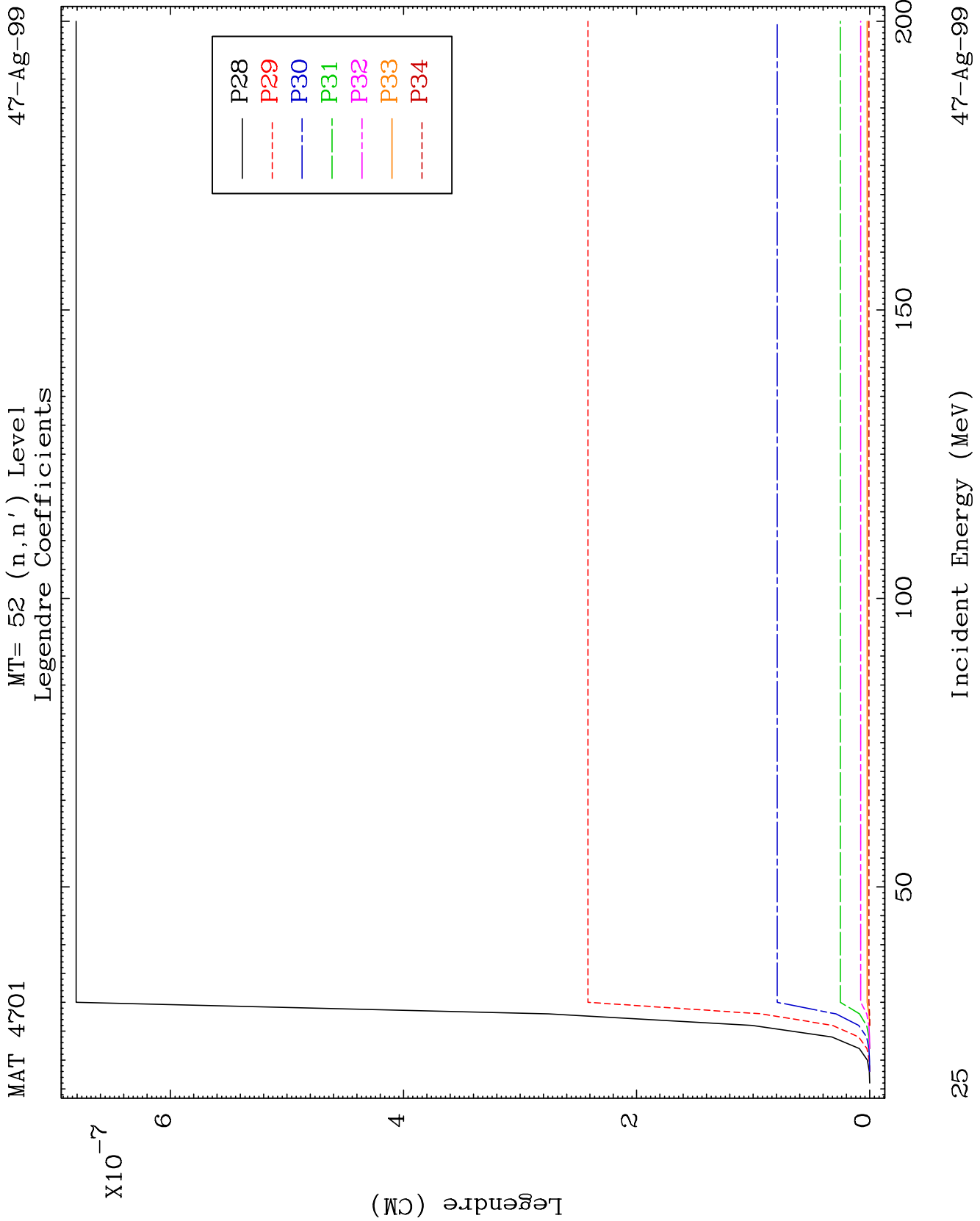


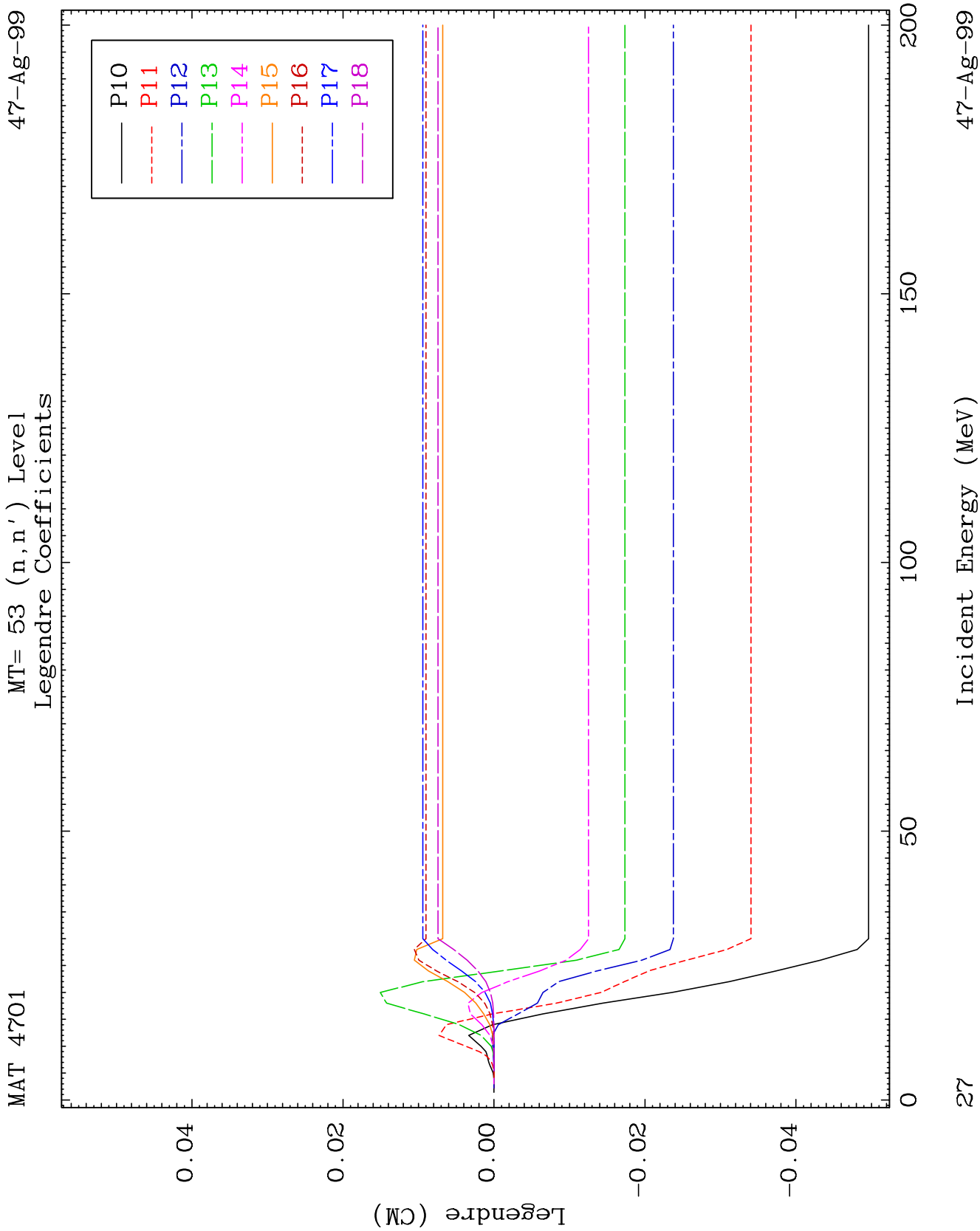
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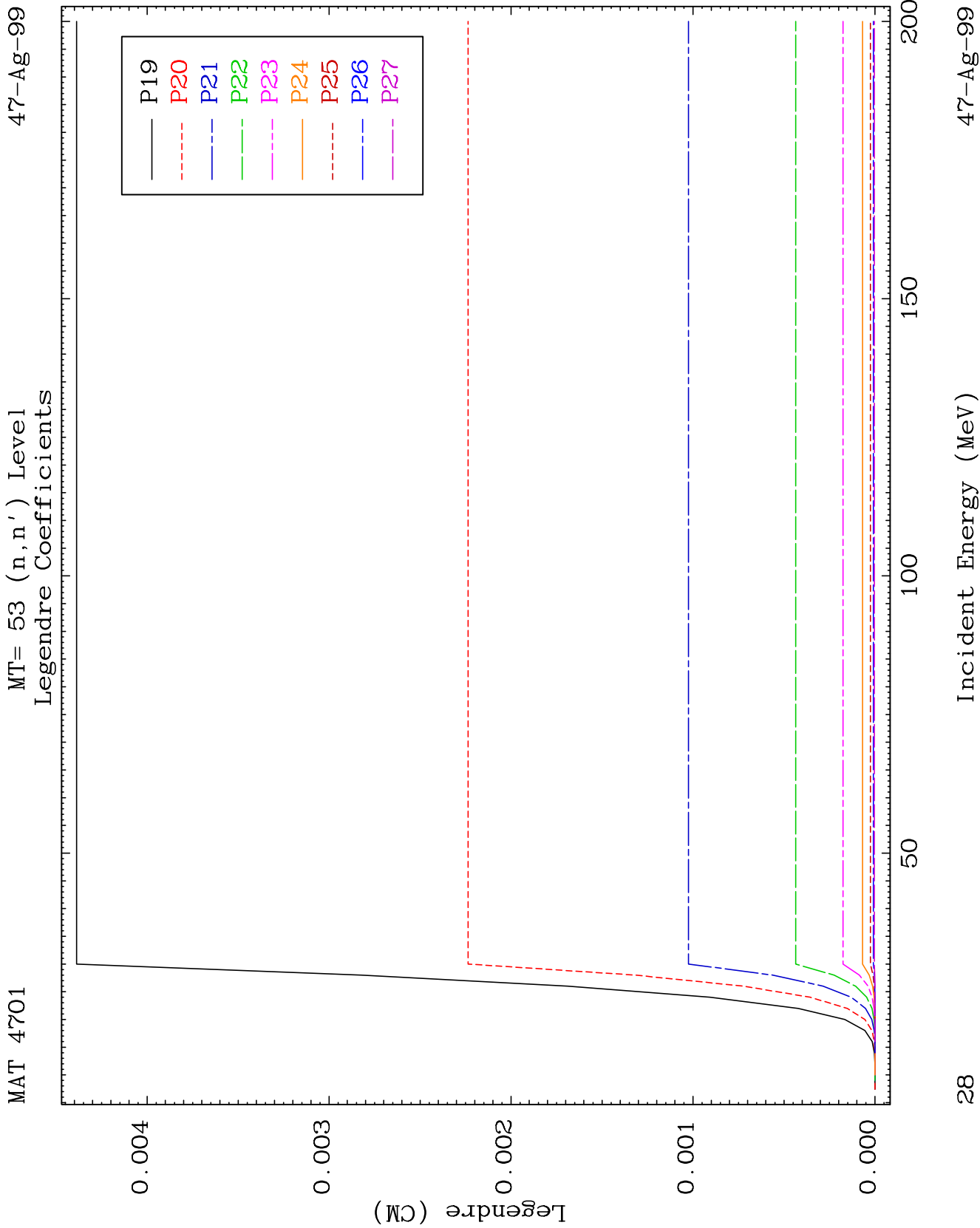
Incident Energy (MeV)

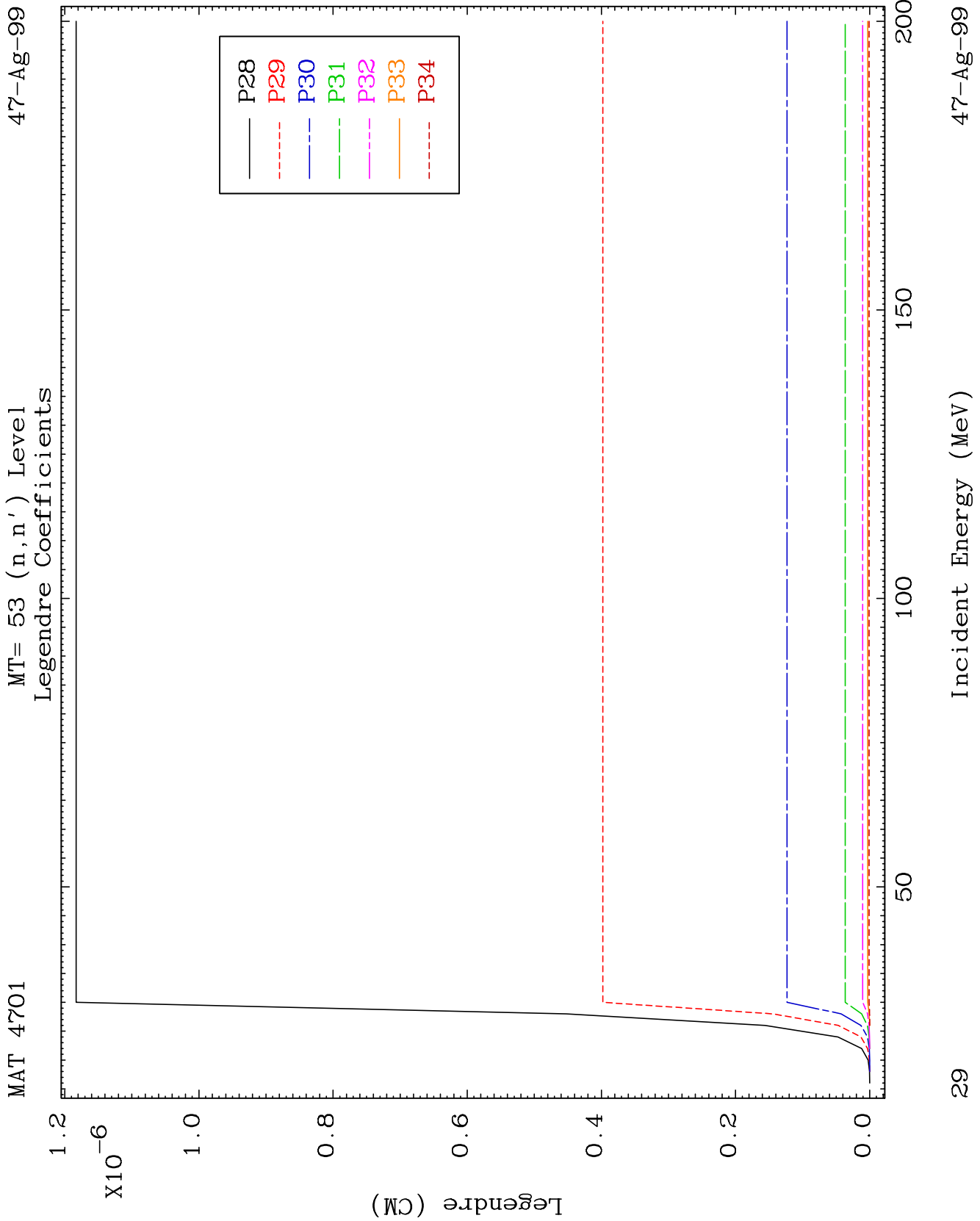
47-A8-99







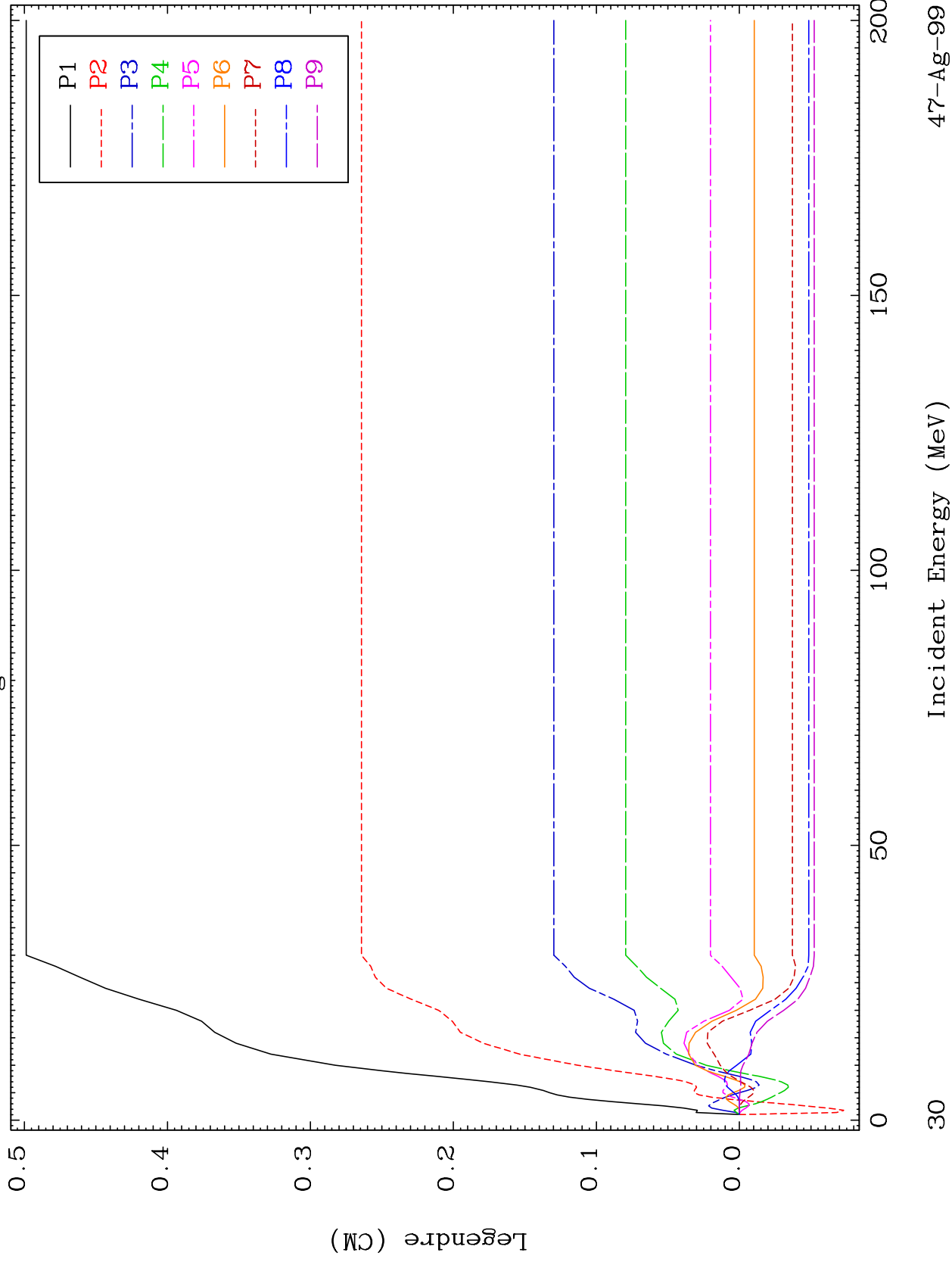


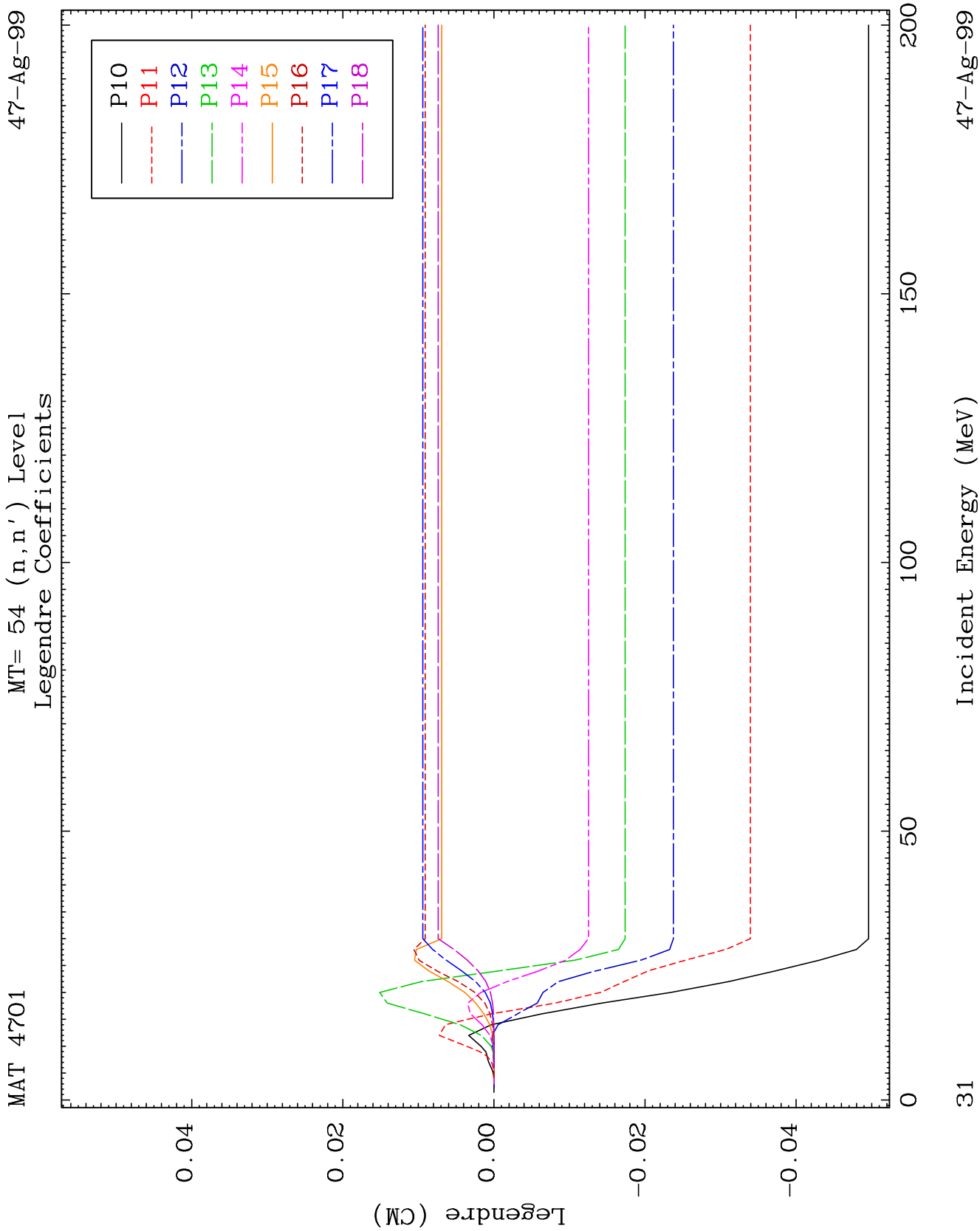


MAT 4701

MT= 54 (n, n') Level
Legendre Coefficients

47-Ag-99

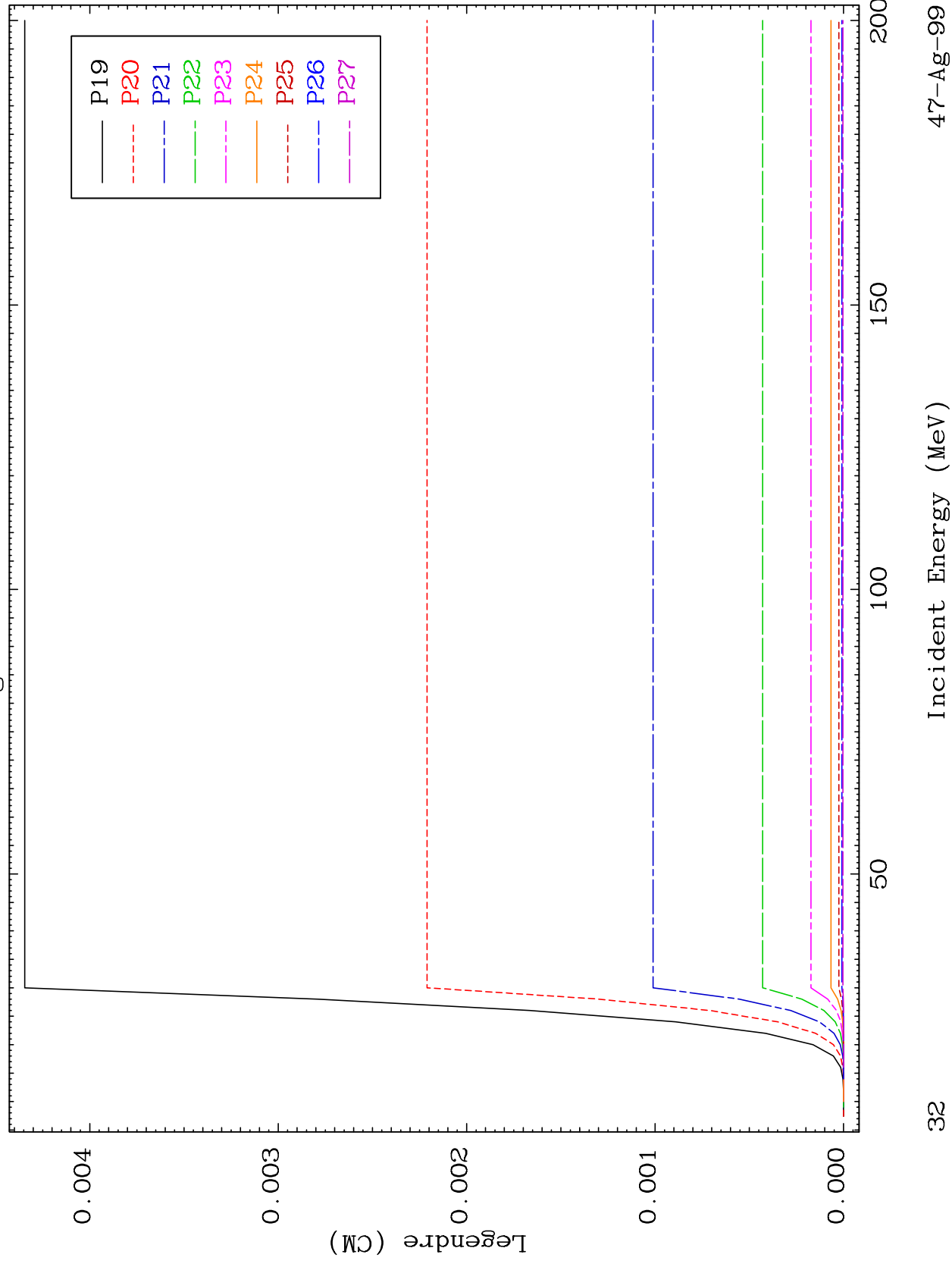


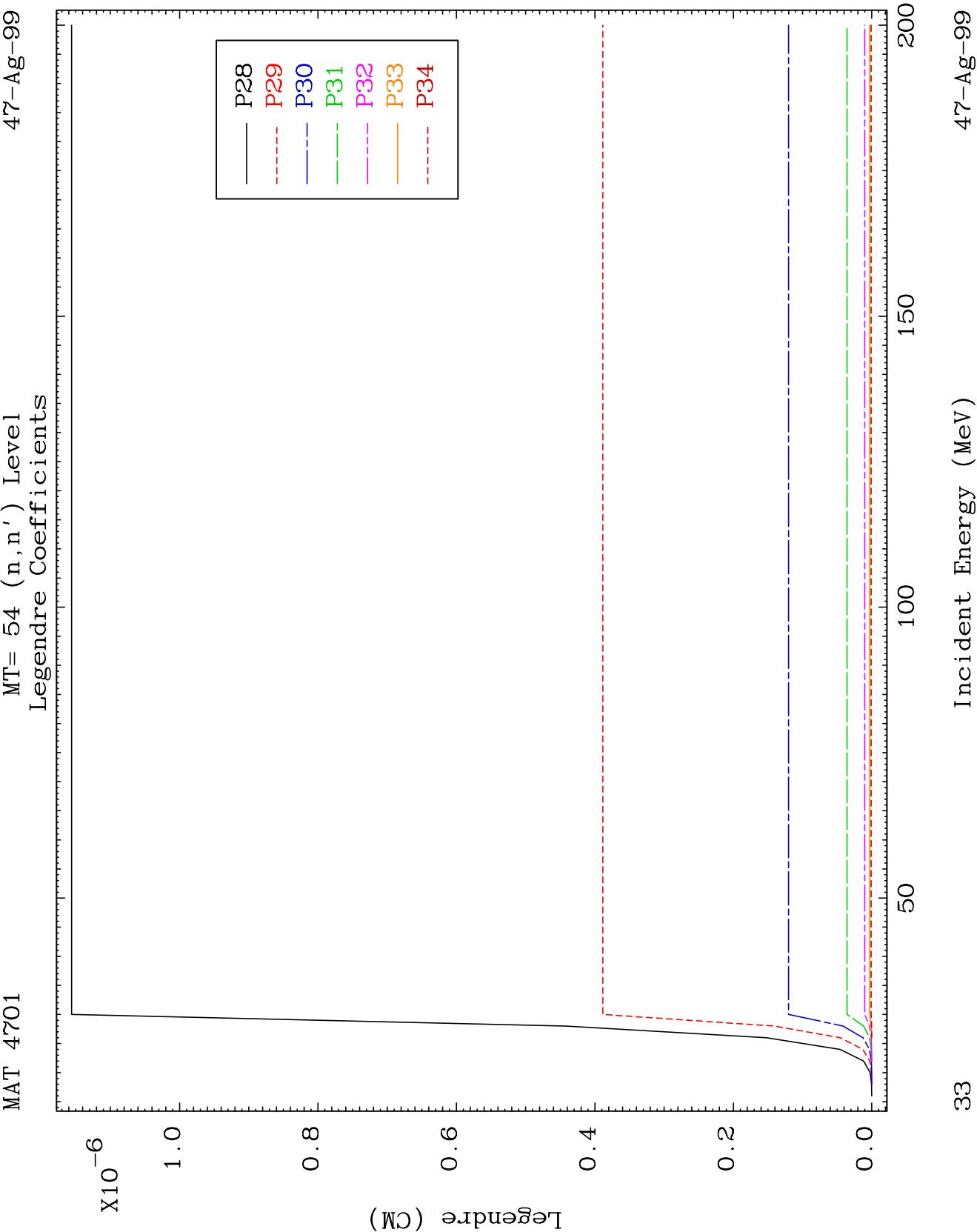


MAT 4701

MT= 54 (n, n') Level
Legendre Coefficients

47-Ag-99

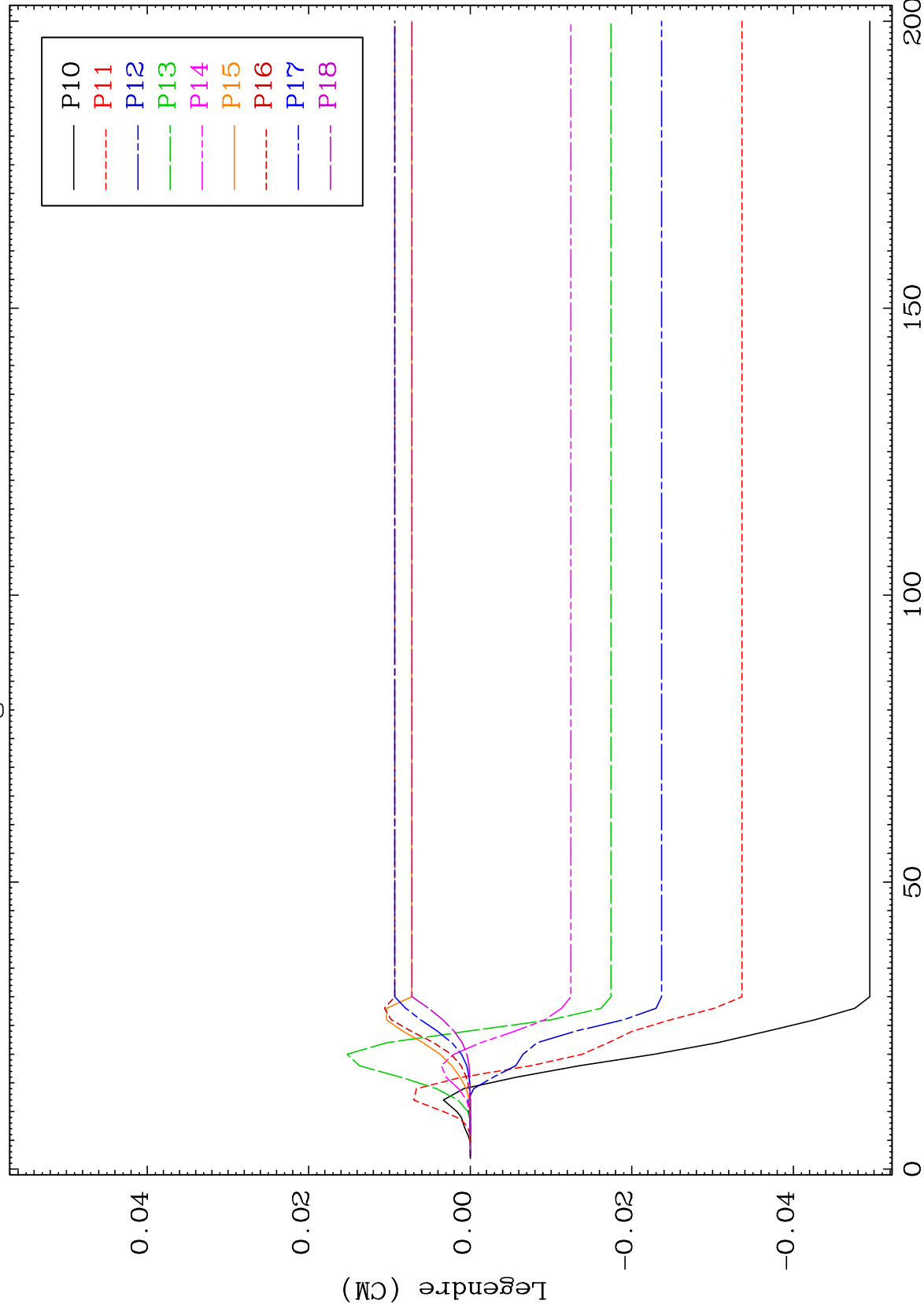




MAT 4701

MT= 55 (n, n') Level
Legendre Coefficients

47-Ag-99



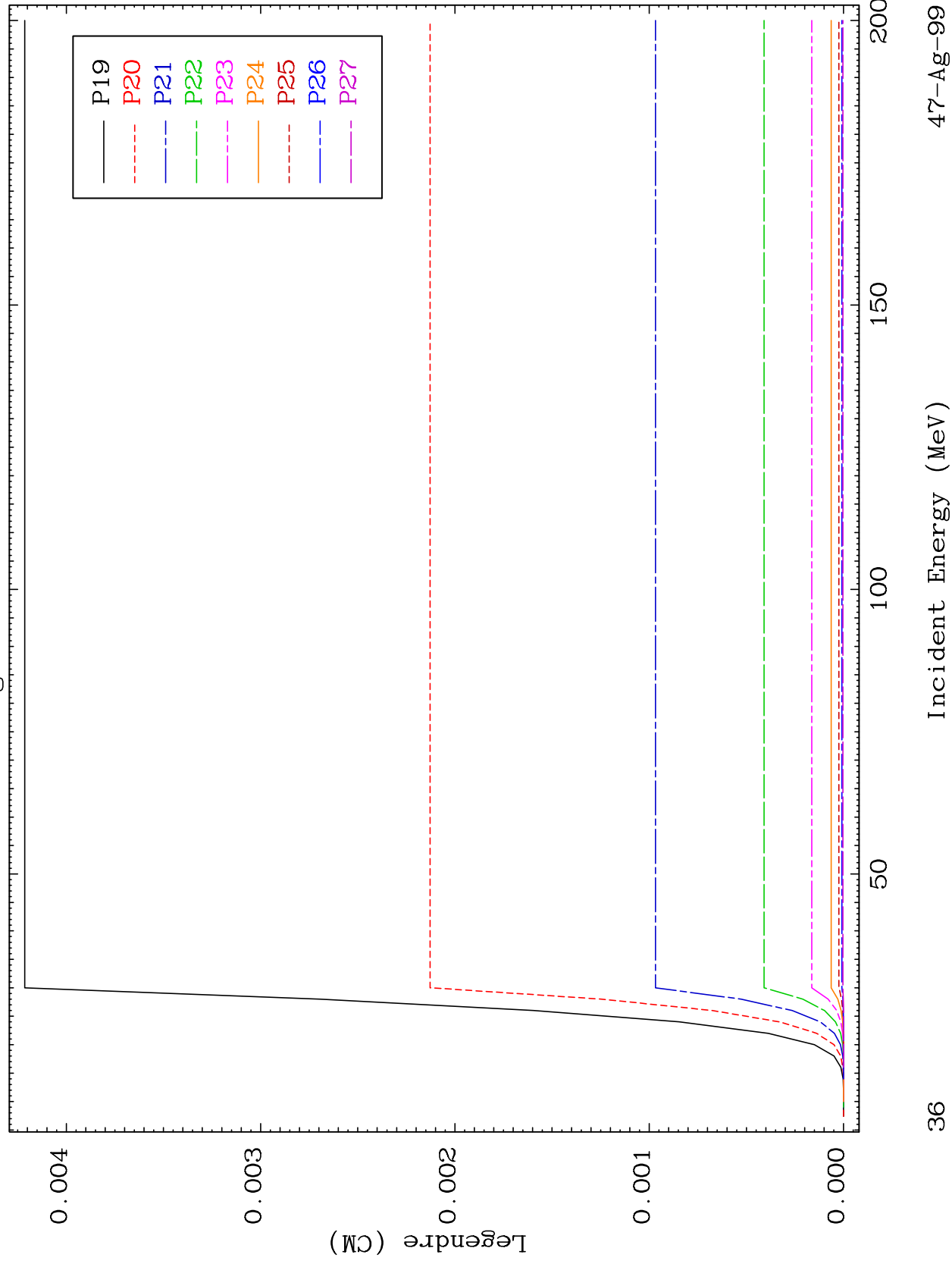
47-A8-99

Incident Energy (MeV)

MAT 4701

MT= 55 (n, n') Level
Legendre Coefficients

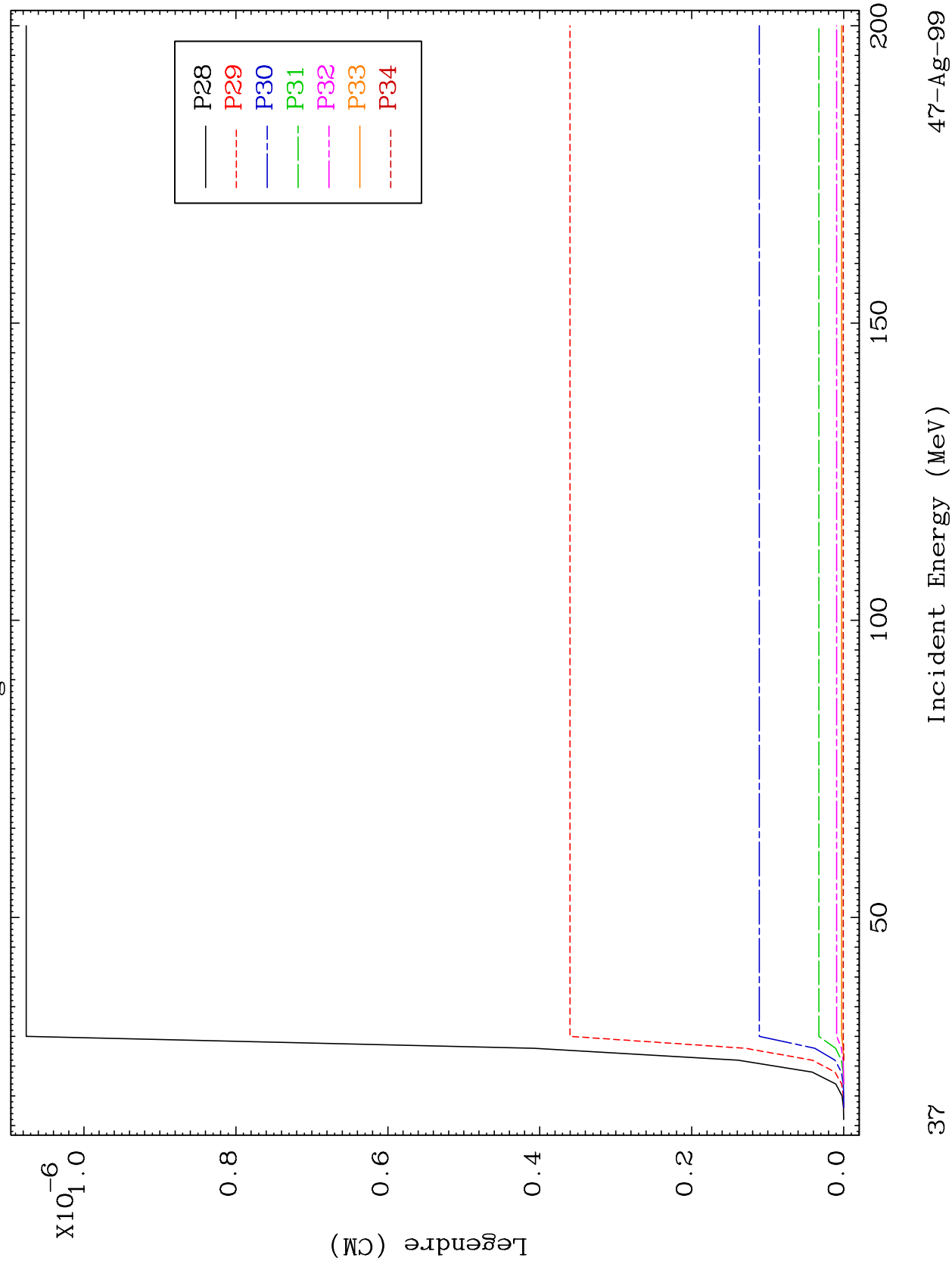
47-Ag-99

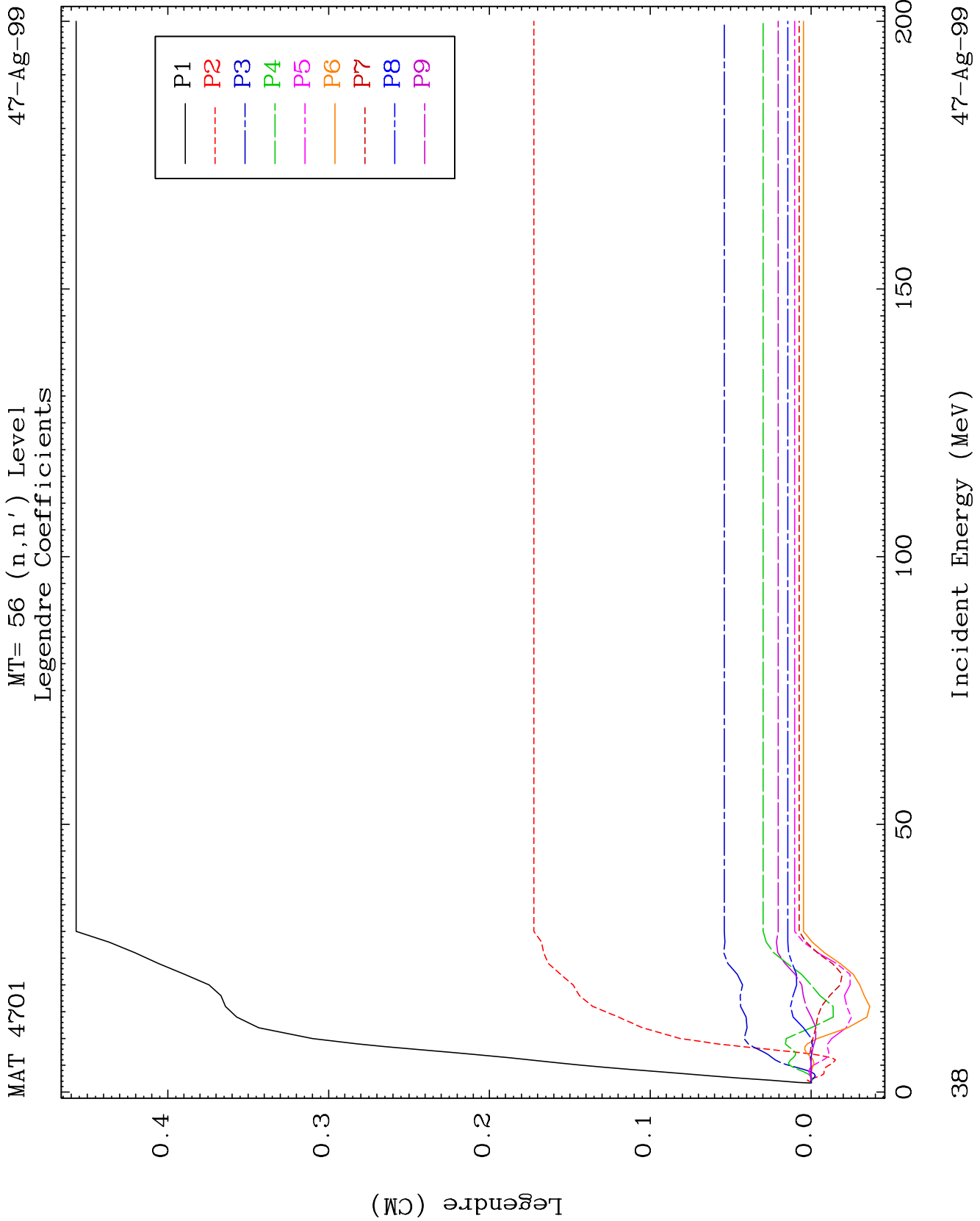


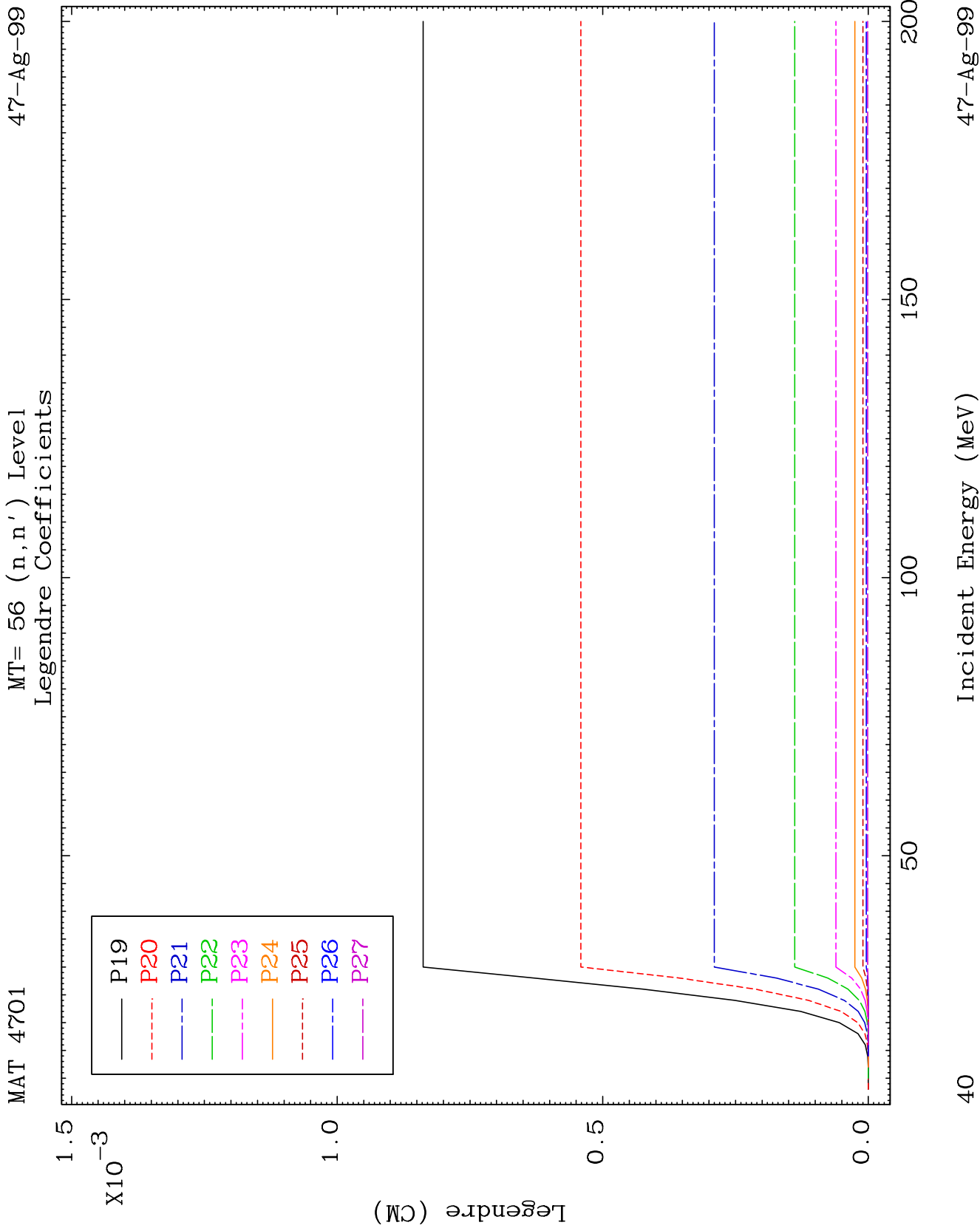
MAT 4701

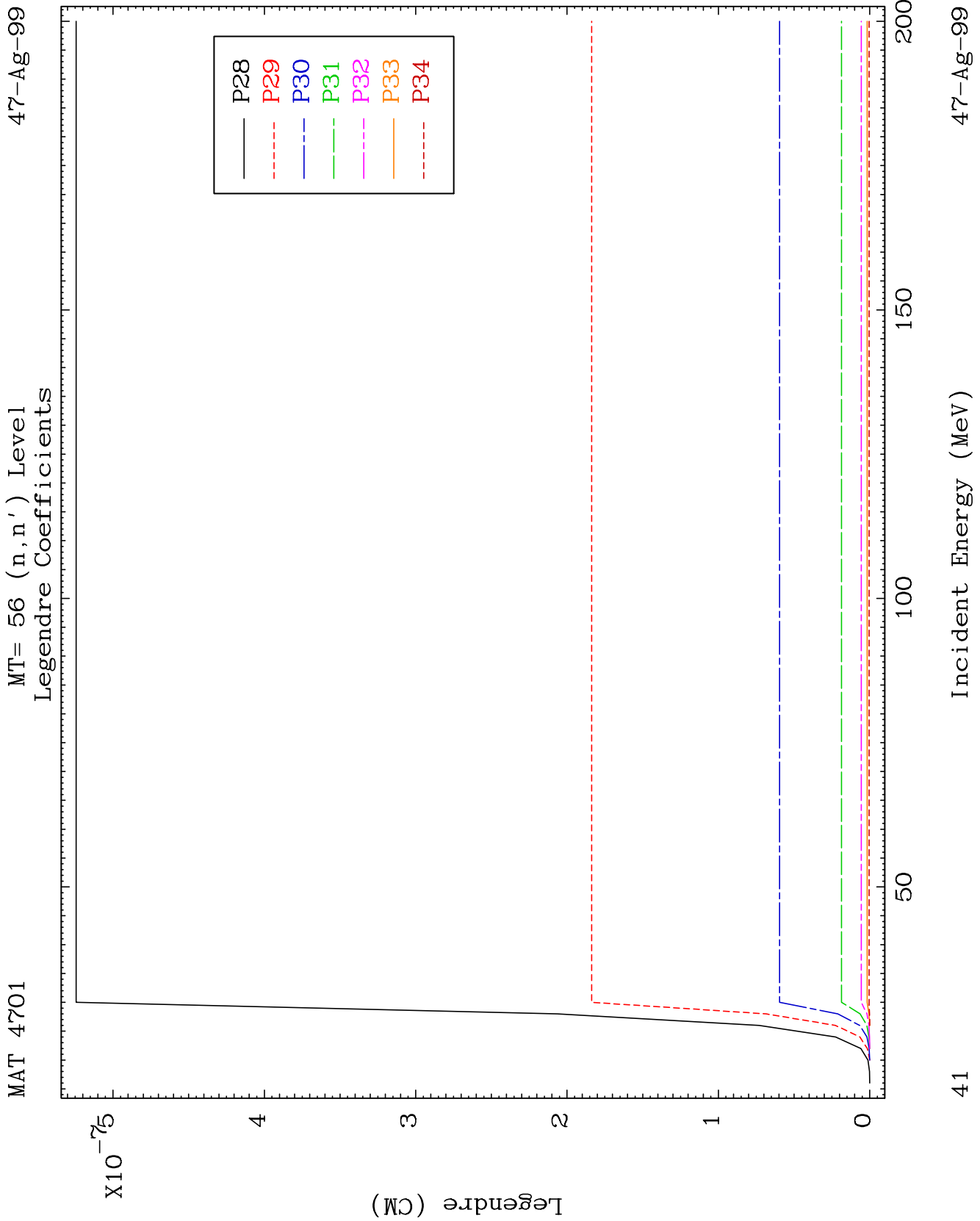
MT= 55 (n, n') Level
Legendre Coefficients

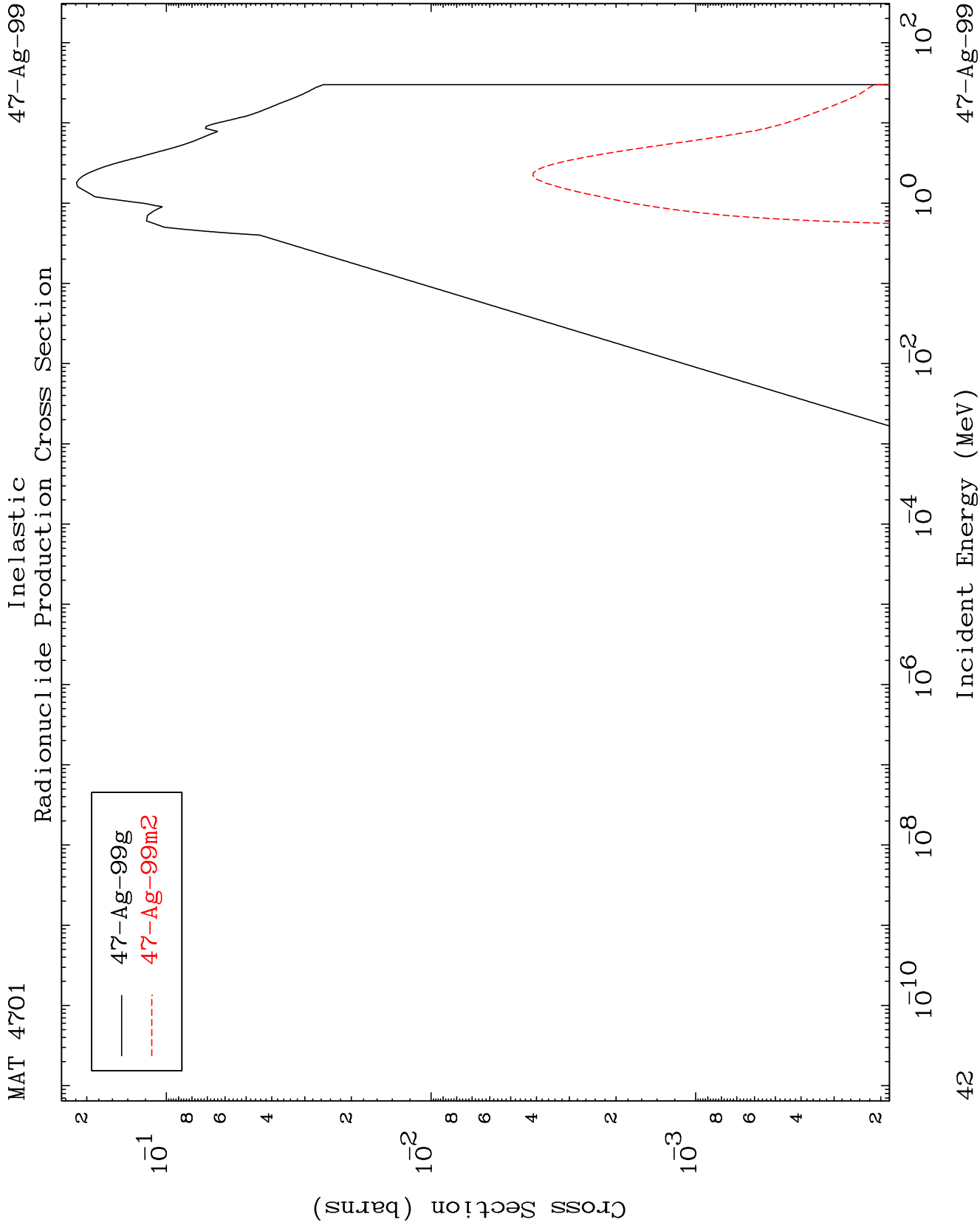
47-Ag-99

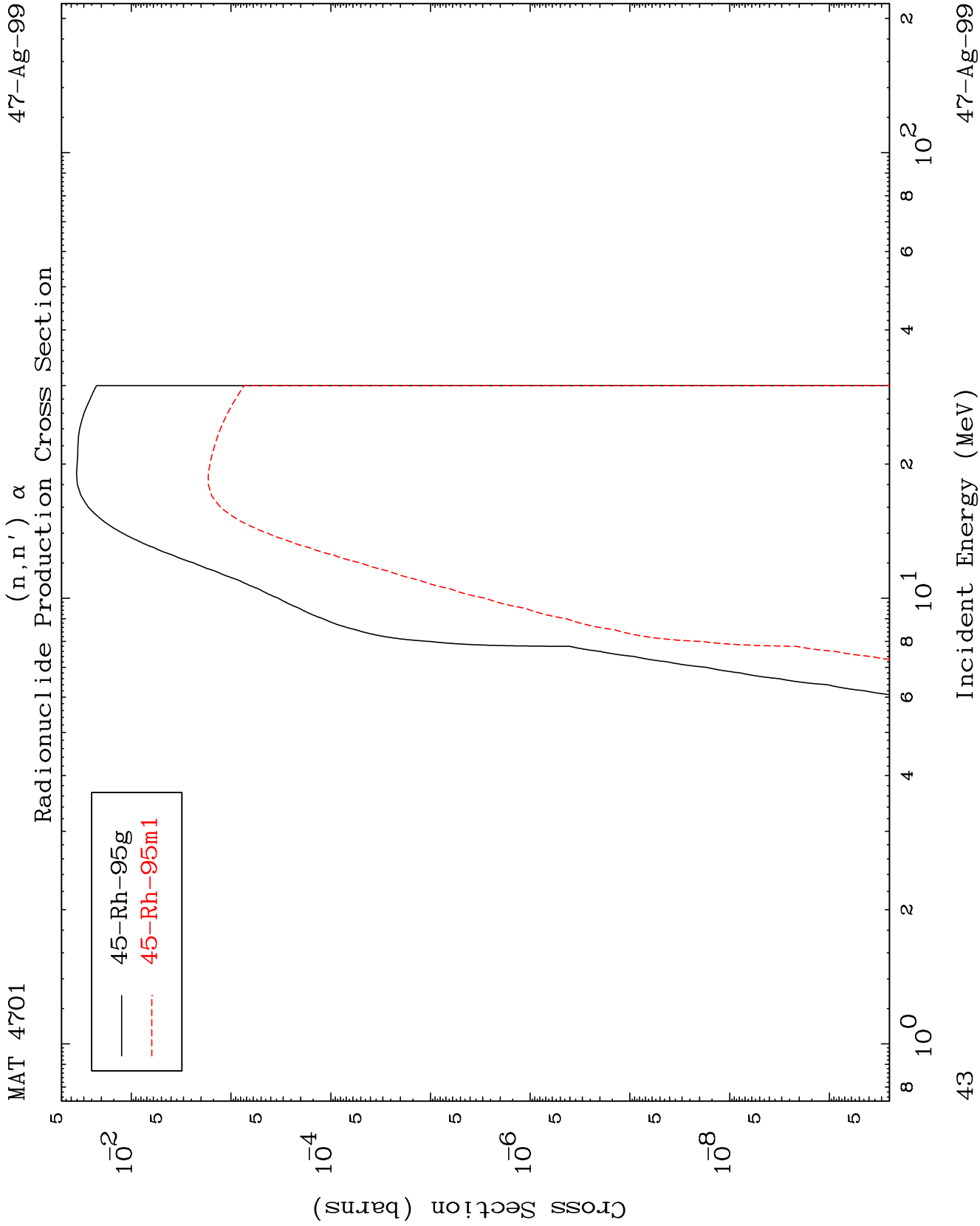








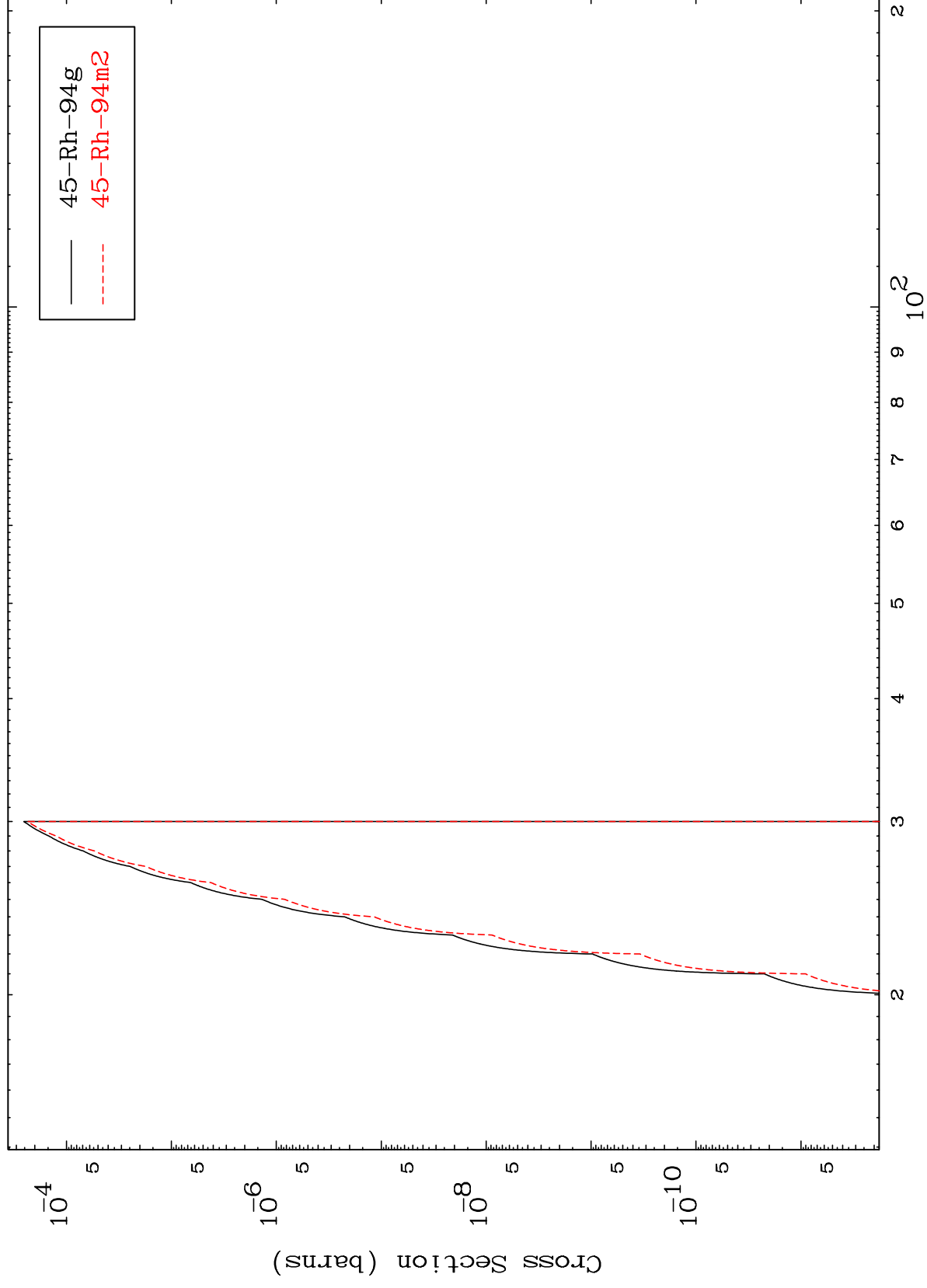




MAT 4701

47-Ag-99

(n,2n) α
Radionuclide Production Cross Section



44

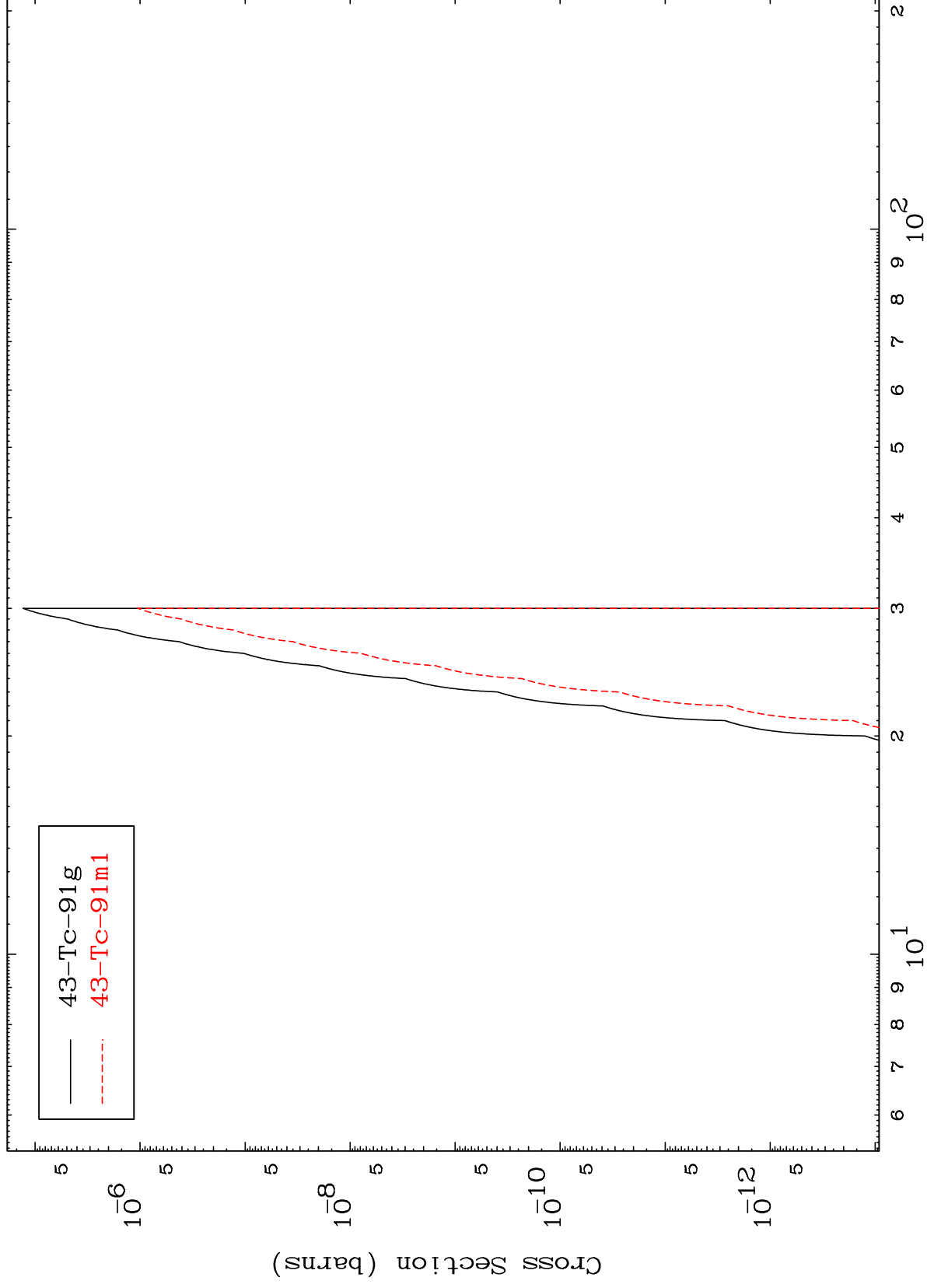
Incident Energy (MeV)

47-Ag-99

MAT 4701

47-Ag-99

$(n,n')\ 2\alpha$
Radionuclide Production Cross Section



45

Incident Energy (MeV)

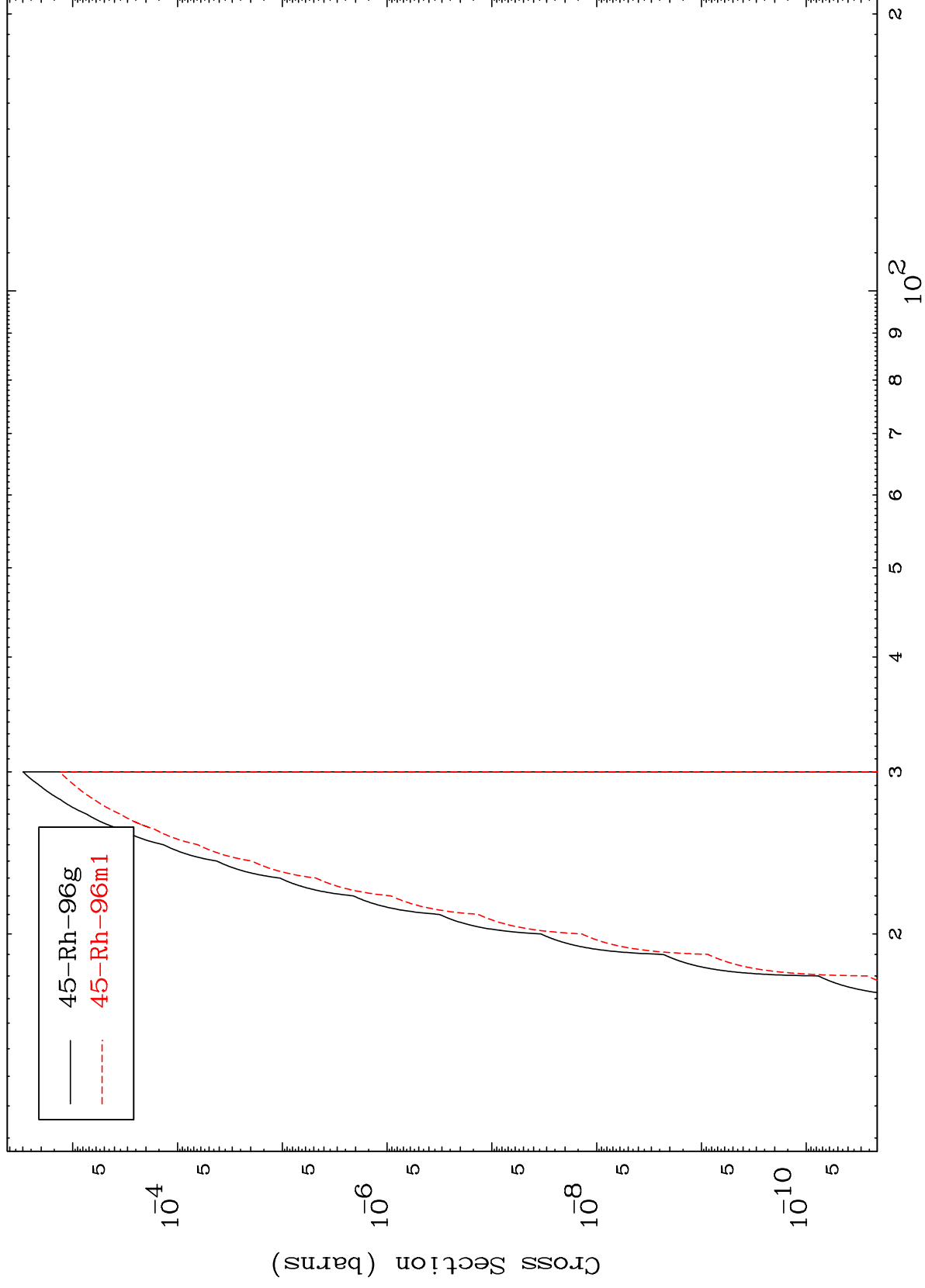
47-Ag-99

MAT 4701

(n,n') He-3

47-Ag-99

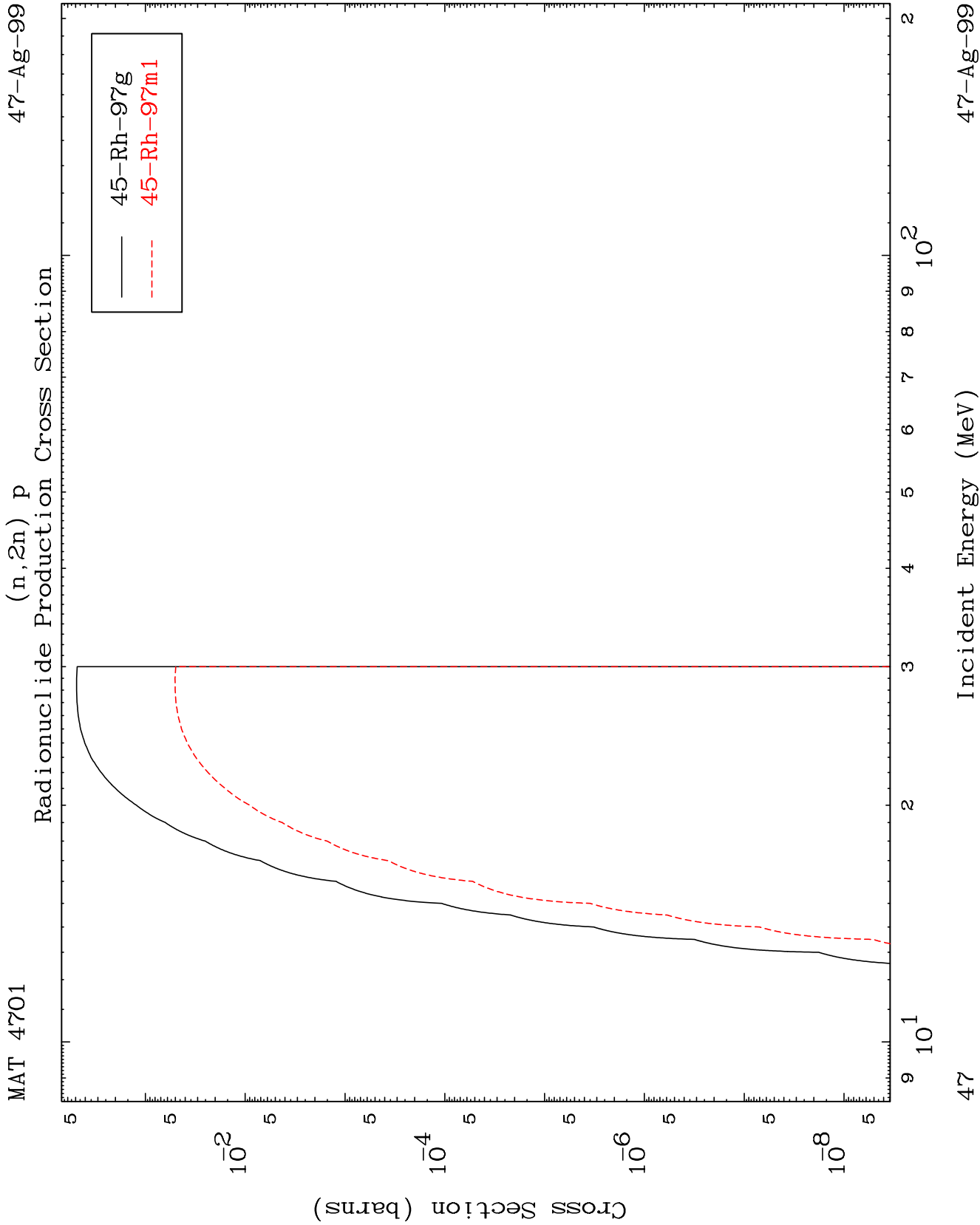
Radionuclide Production Cross Section

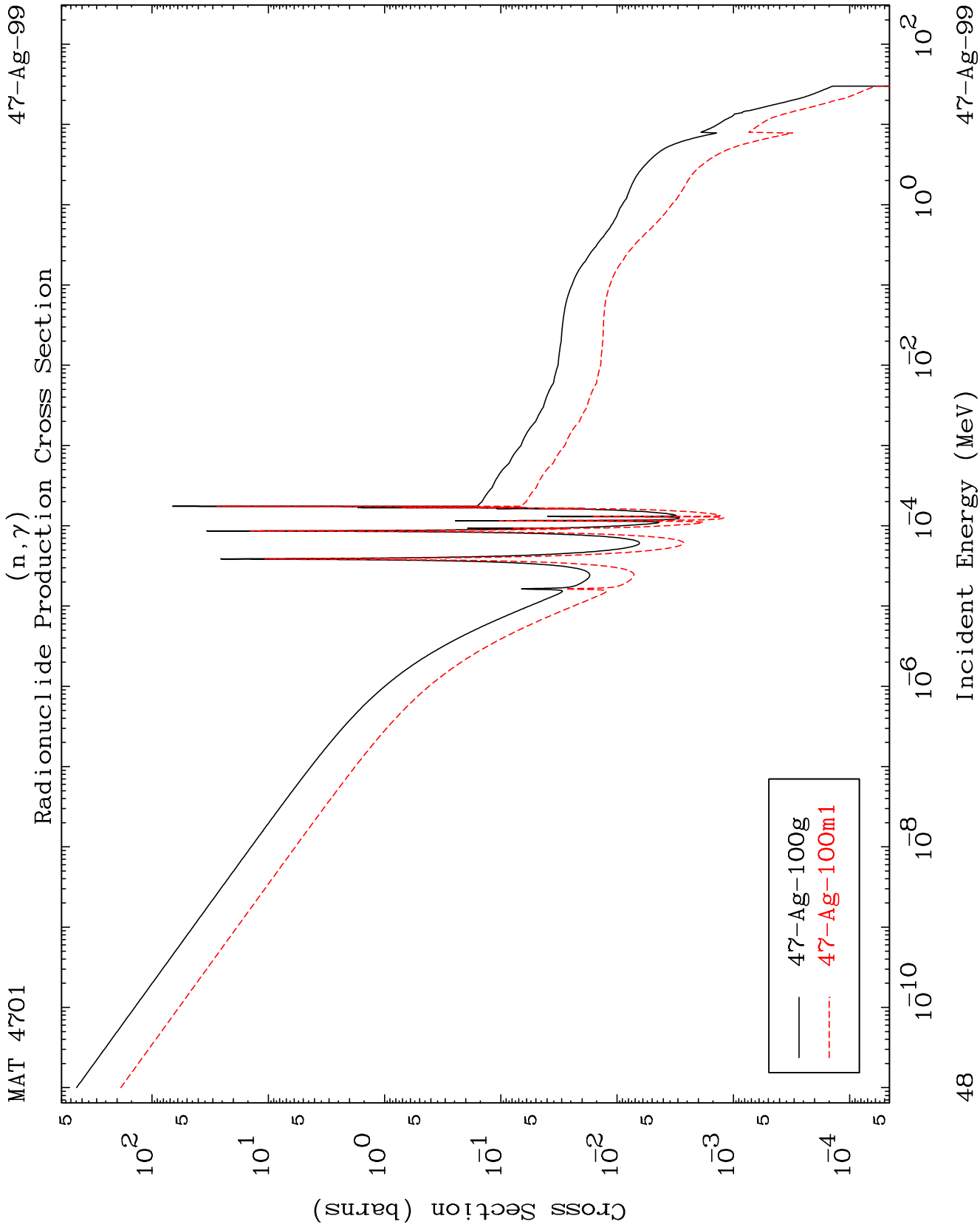


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Incident Energy (MeV)

47-Ag-99

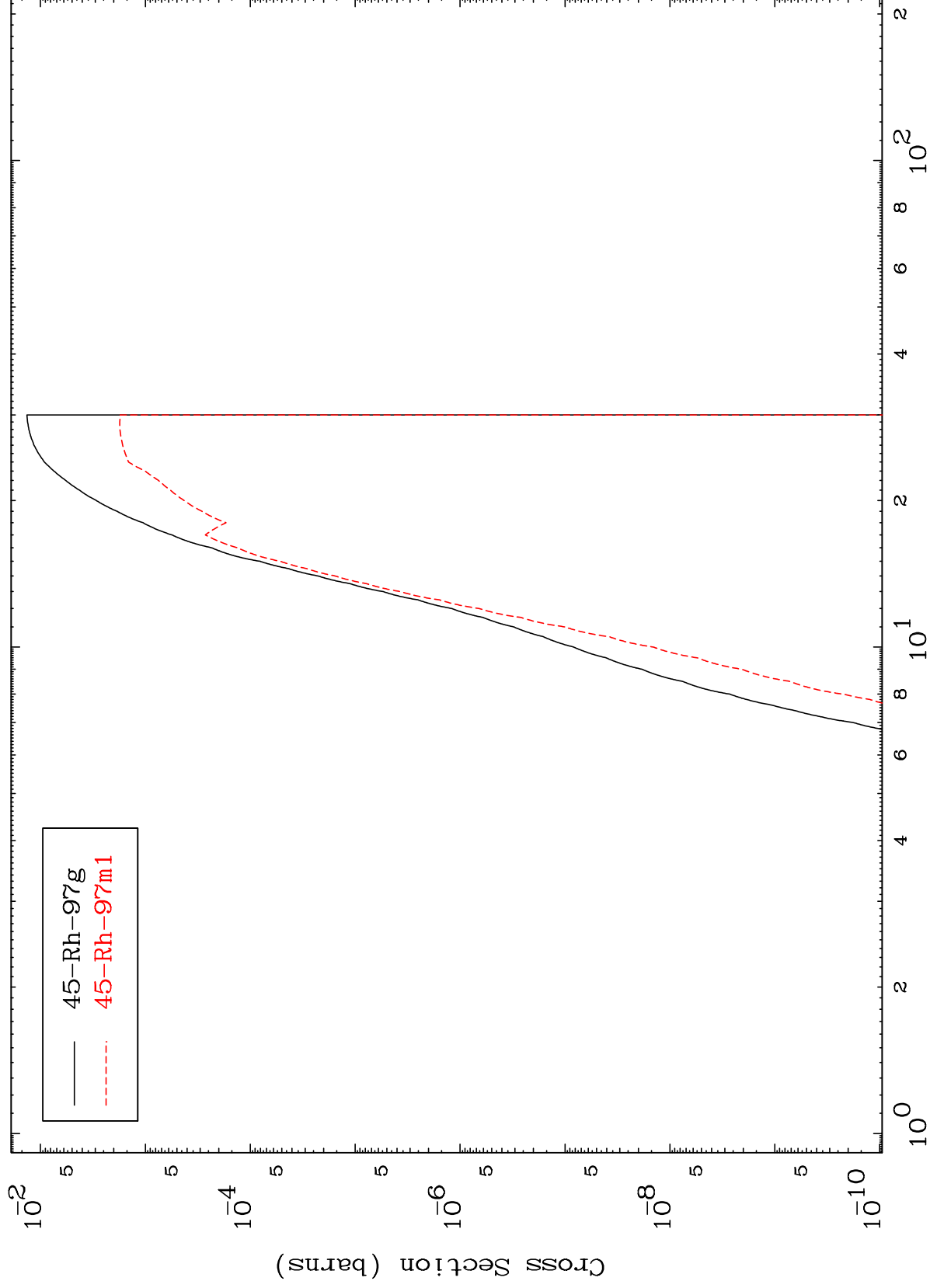




MAT 4701

47-Ag-99

Radionuclide Production Cross Section



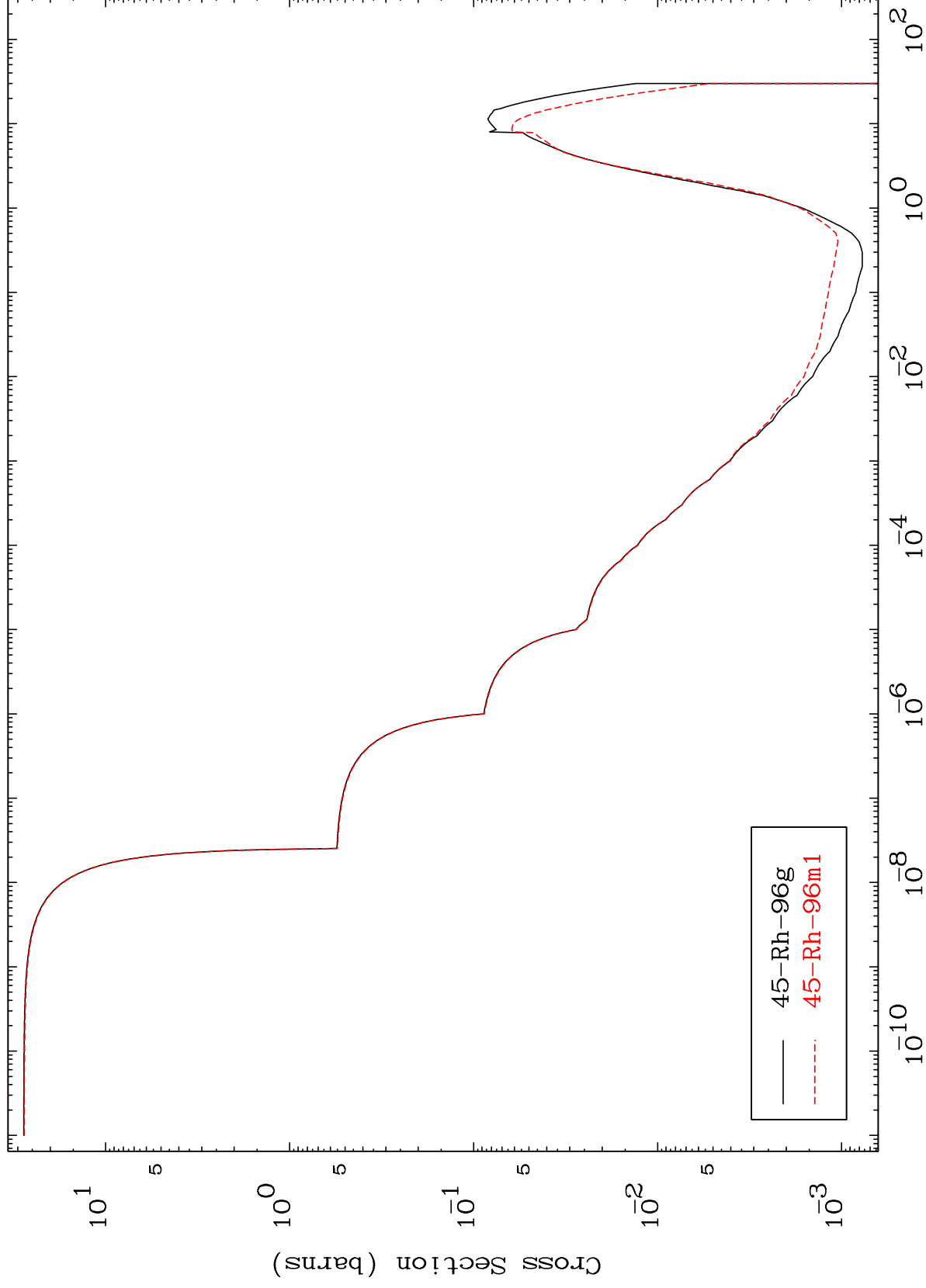
Incident Energy (MeV)

47-Ag-99

MAT 4701

47-Ag-99

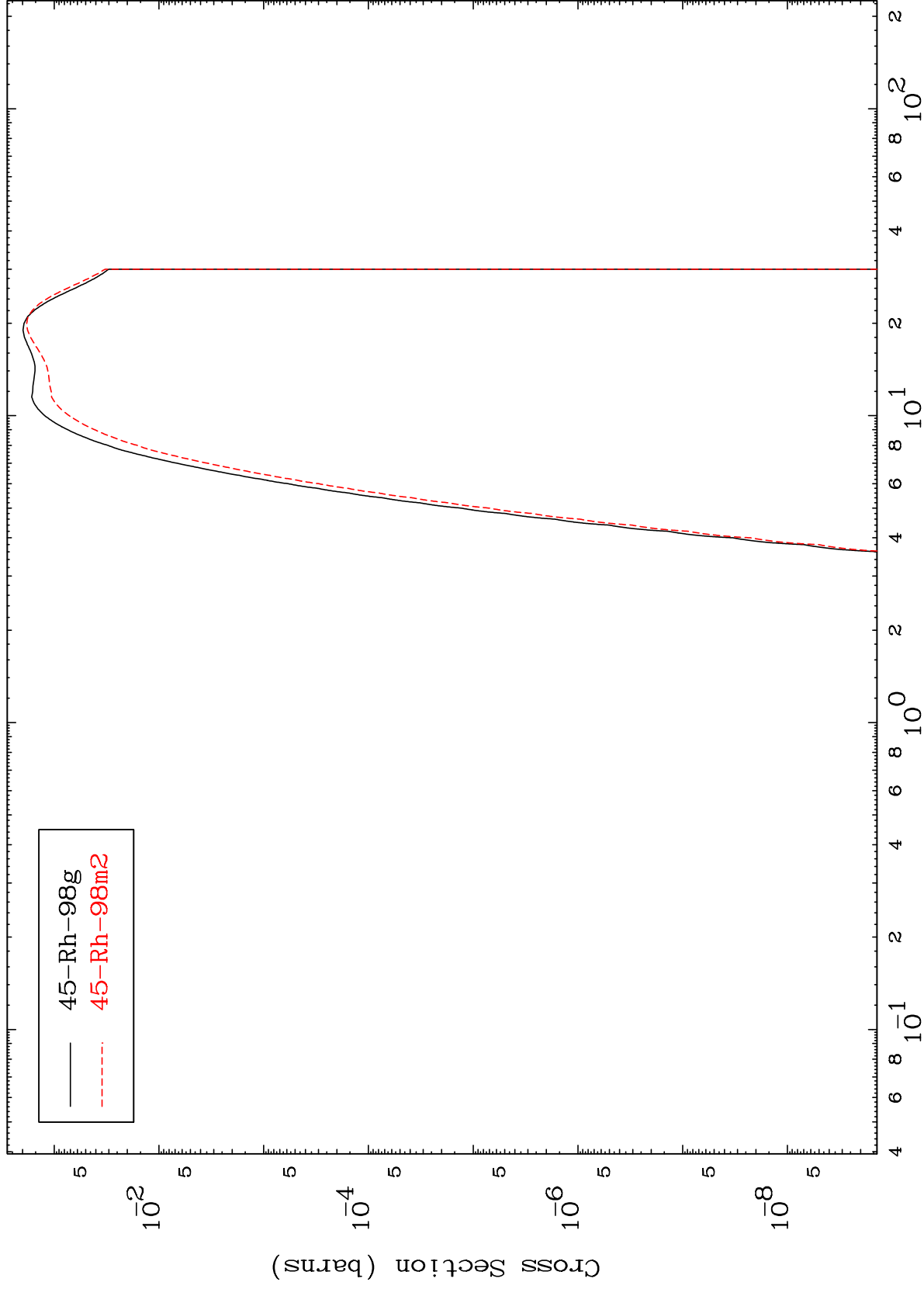
Radionuclide Production Cross Section
(n, α)



50

Incident Energy (MeV)

47-Ag-99

Radionuclide Production Cross Section
(n,2p)

(n,p) d
Radionuclide Production Cross Section