

Program EVALPLOT
(Version 2021-1)

by

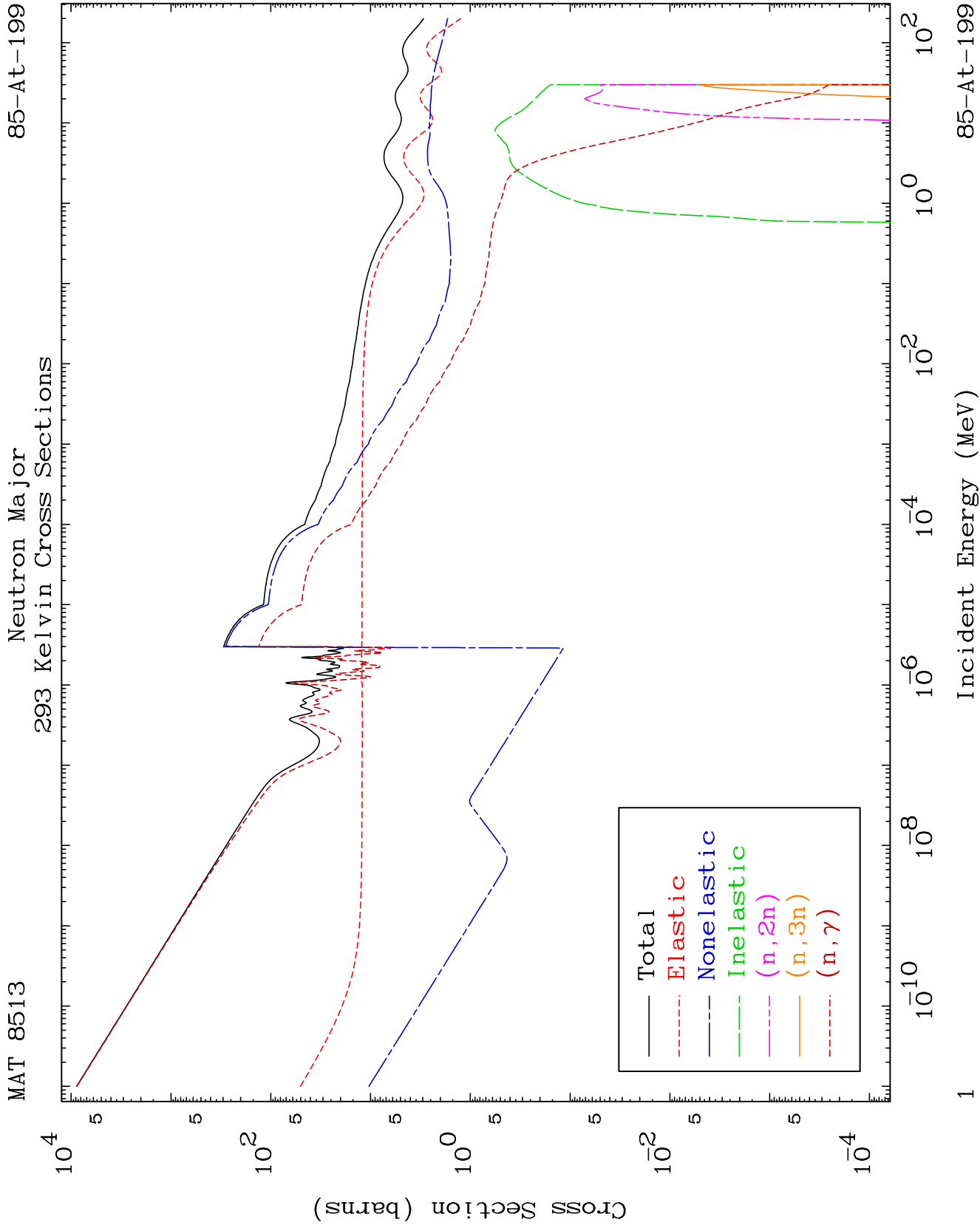
Dermott E. Cullen
(Present Contact Information)

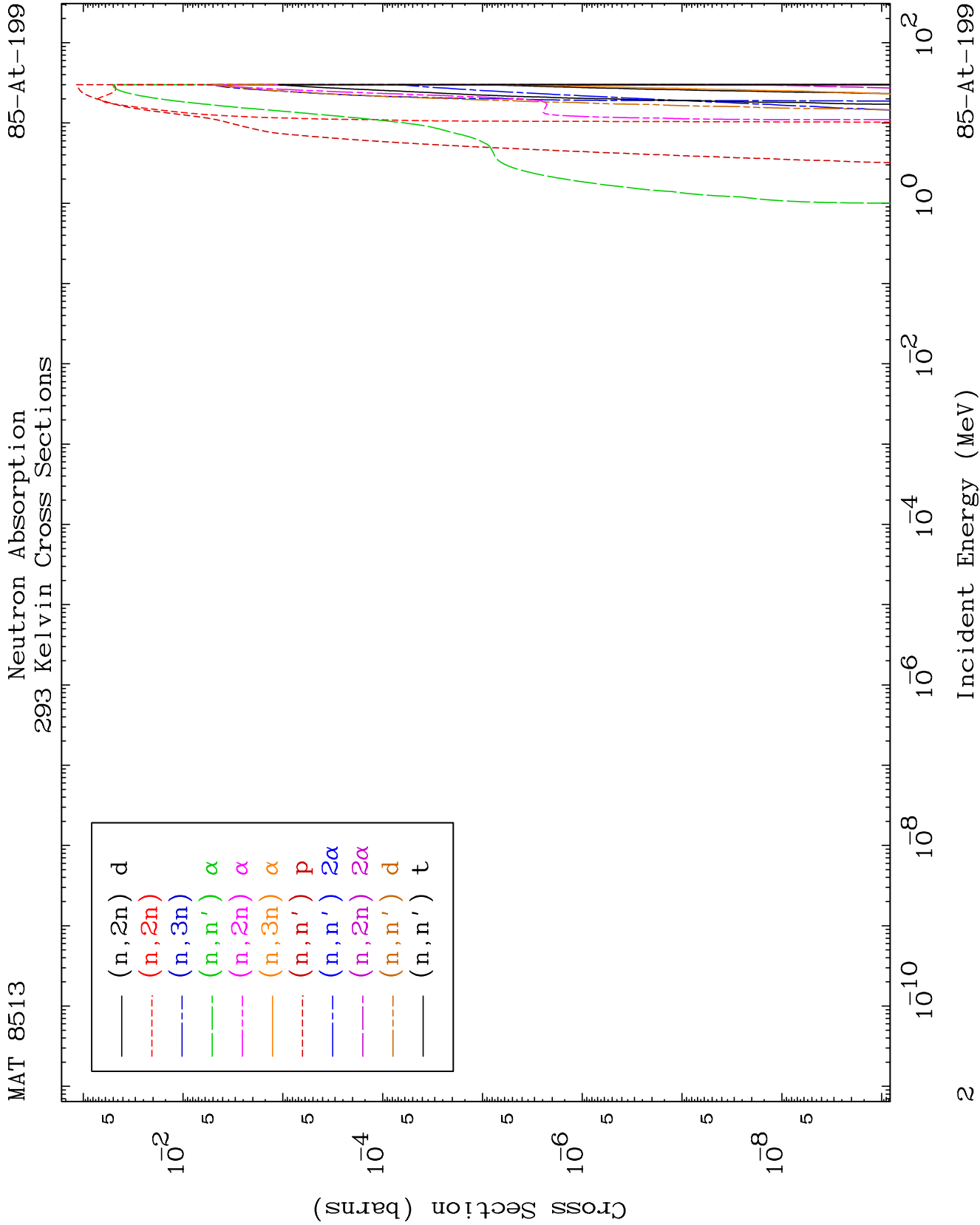
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

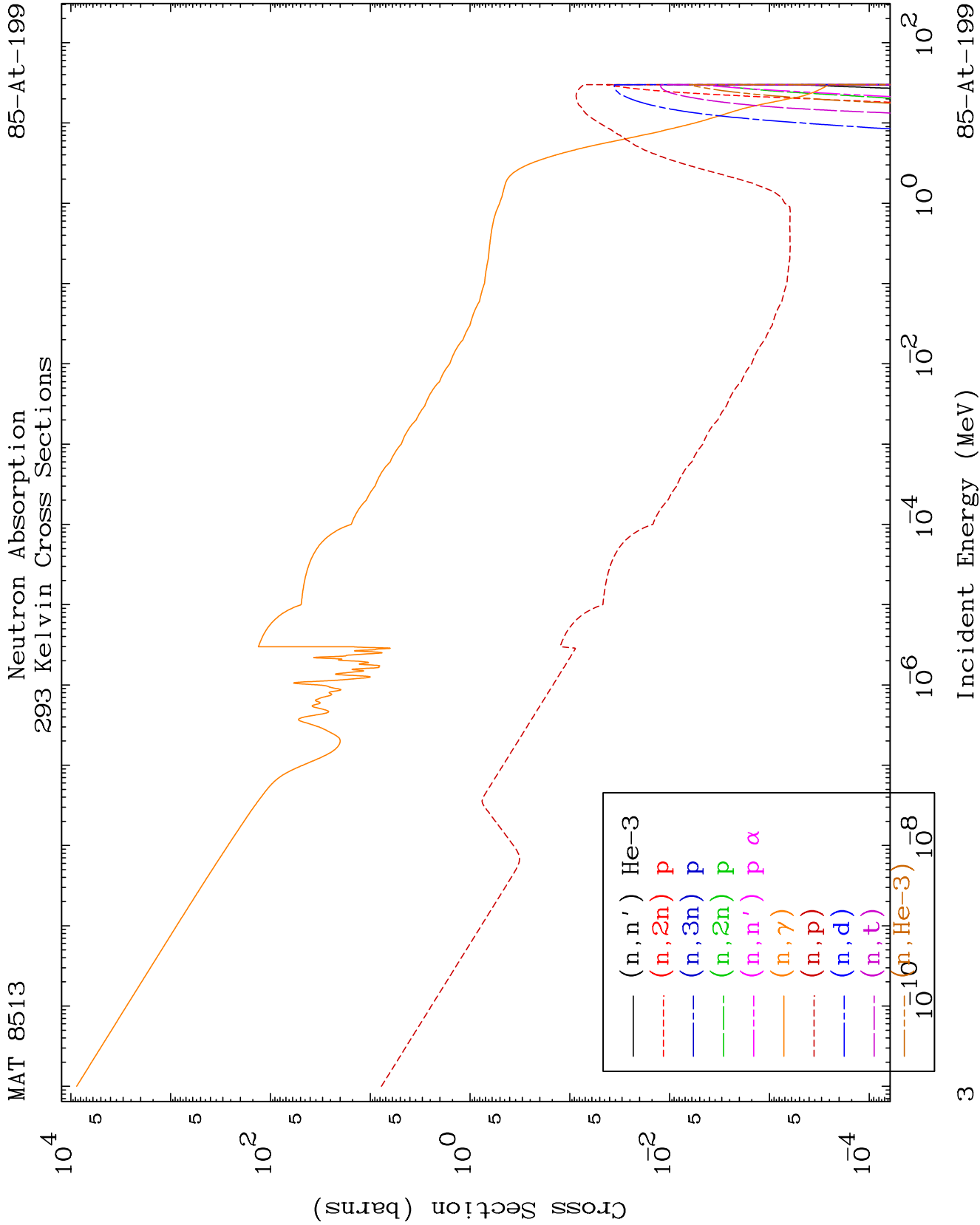
Tele: 925-443-1911

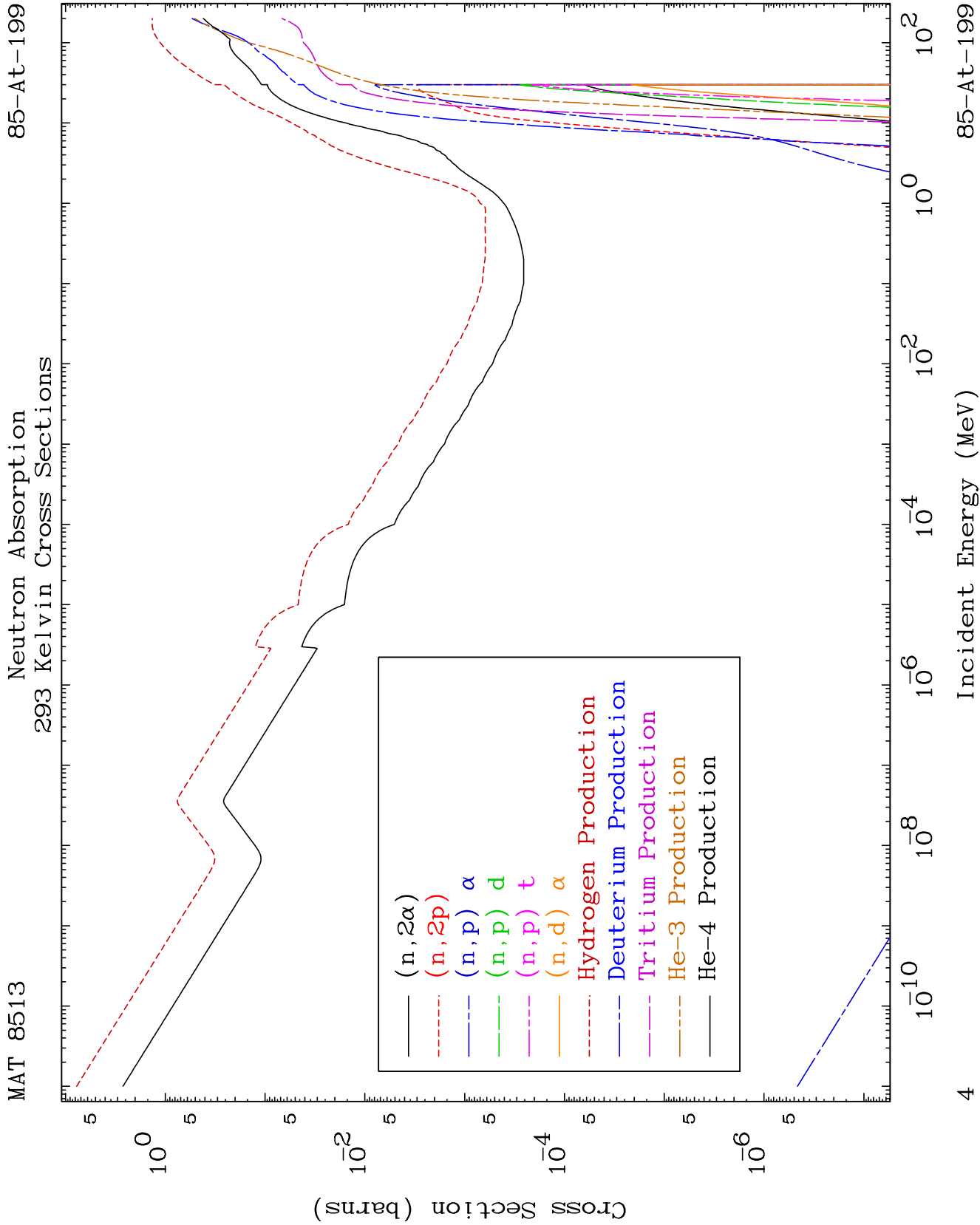
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

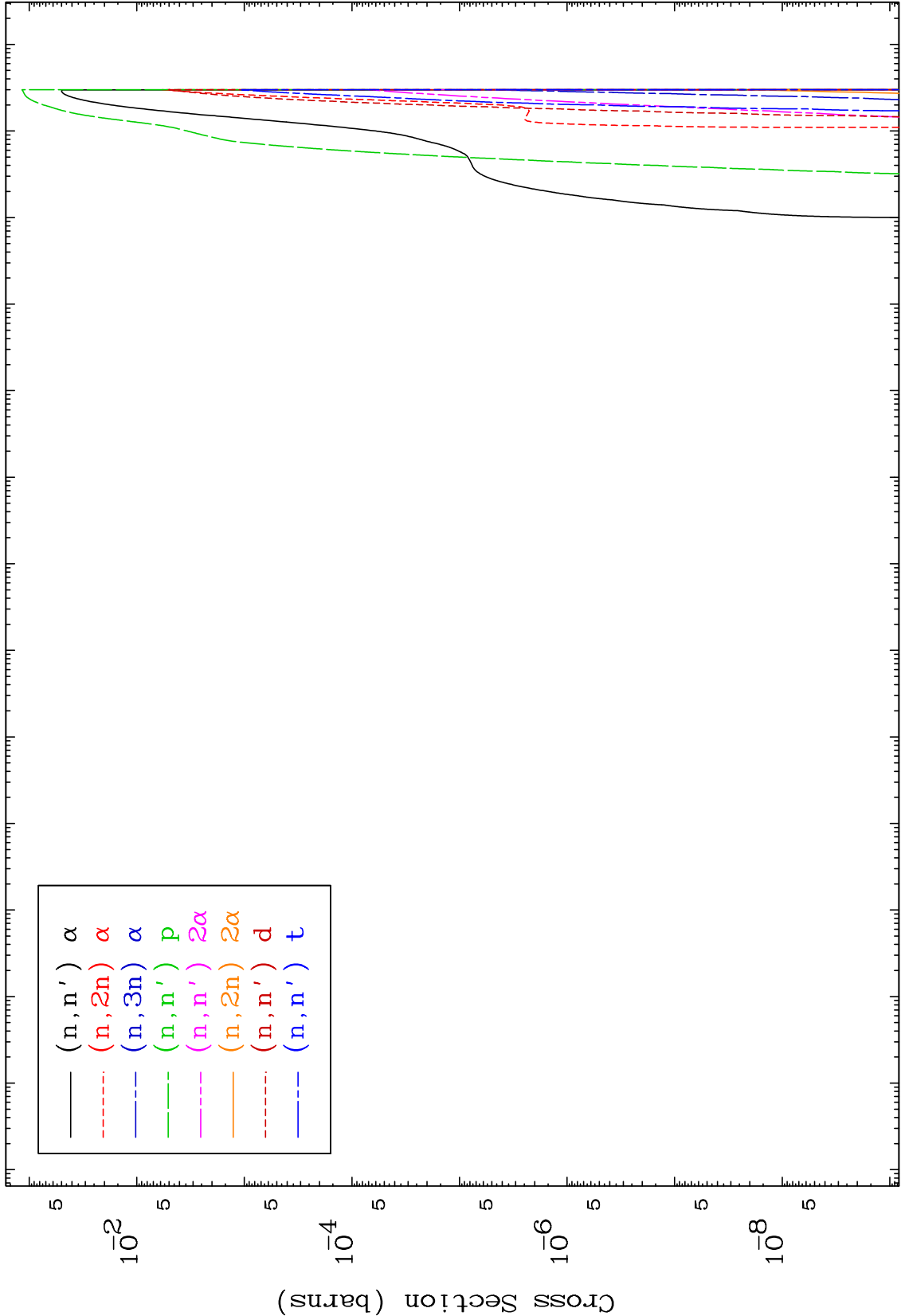
Press Mouse Button to Start

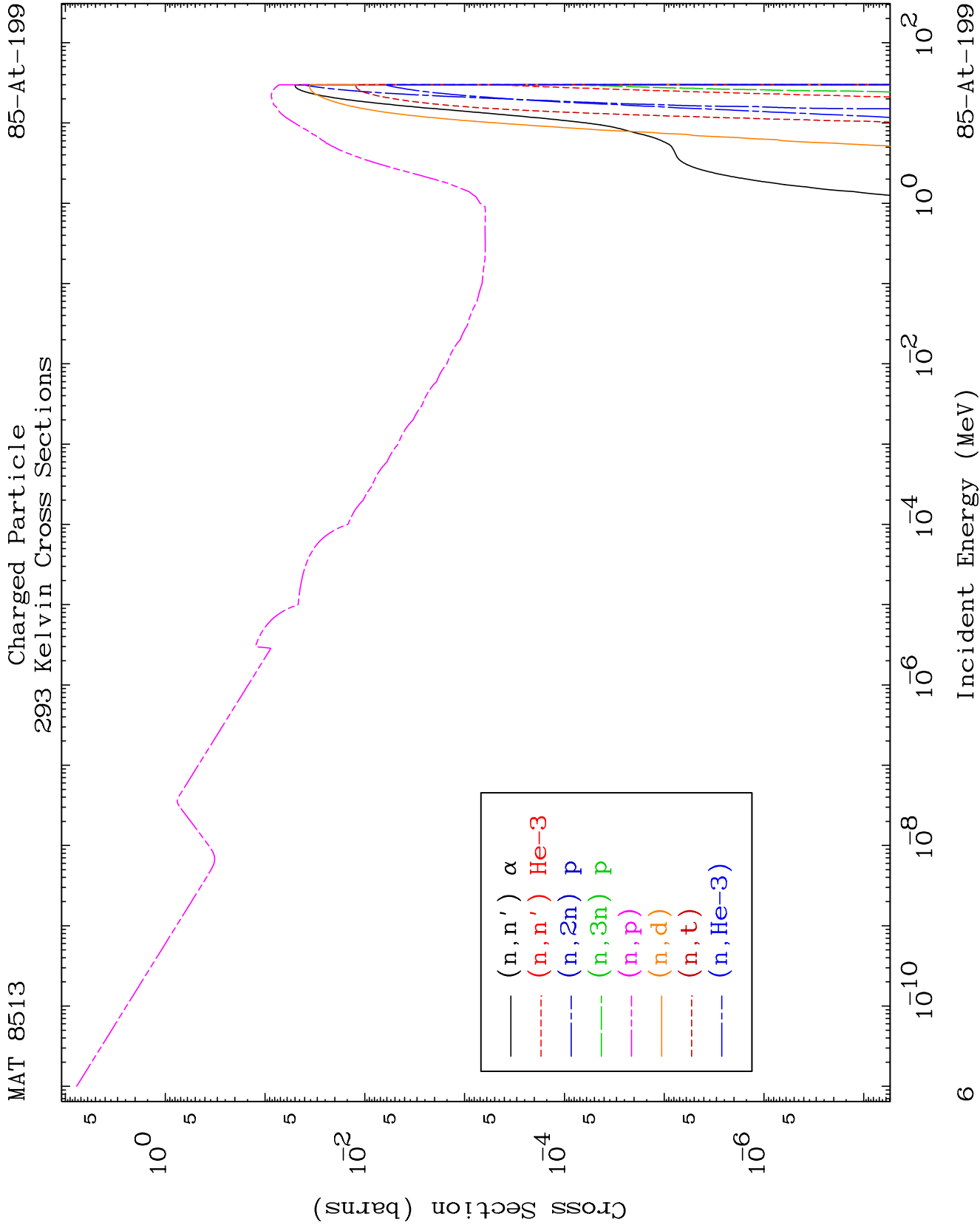


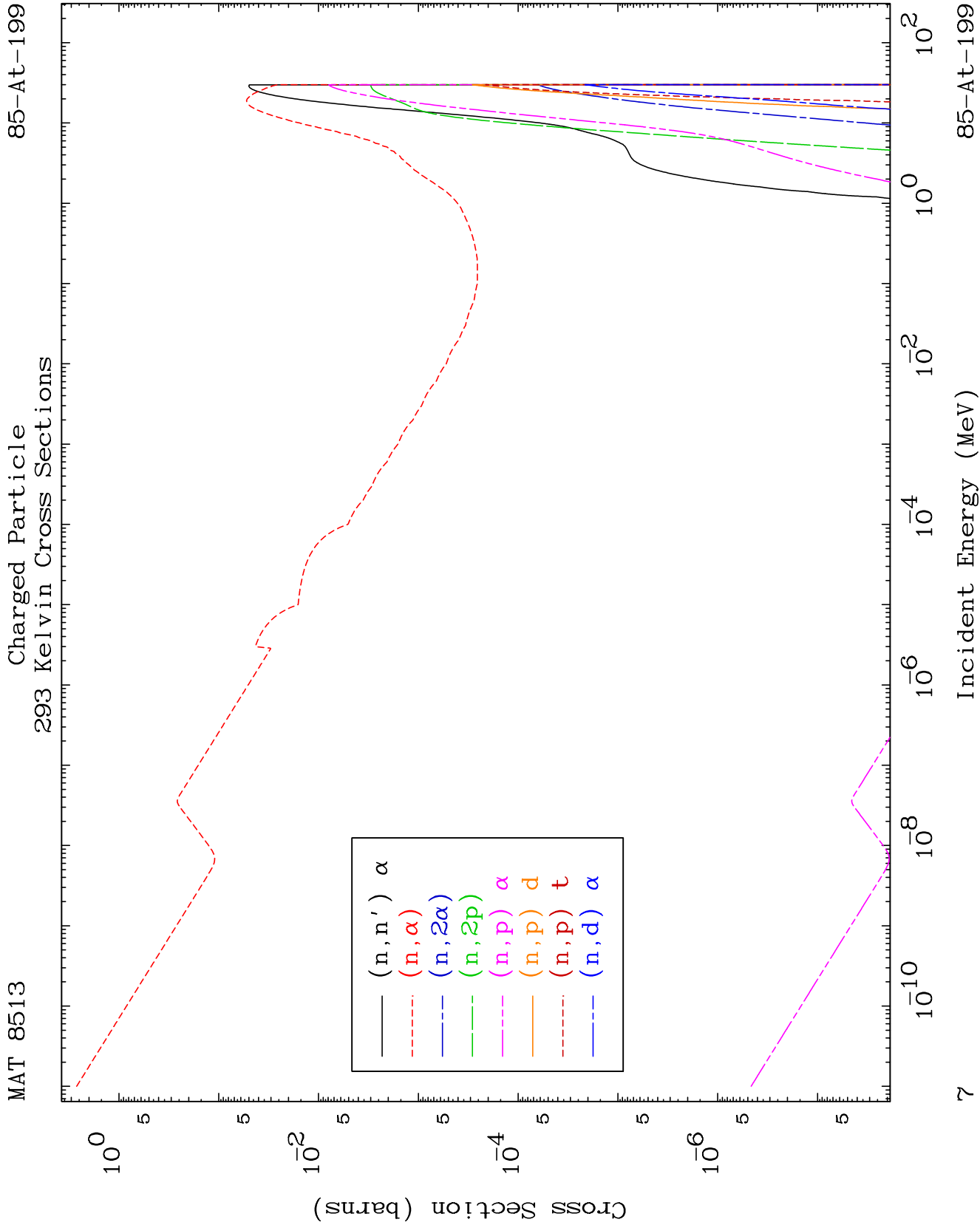


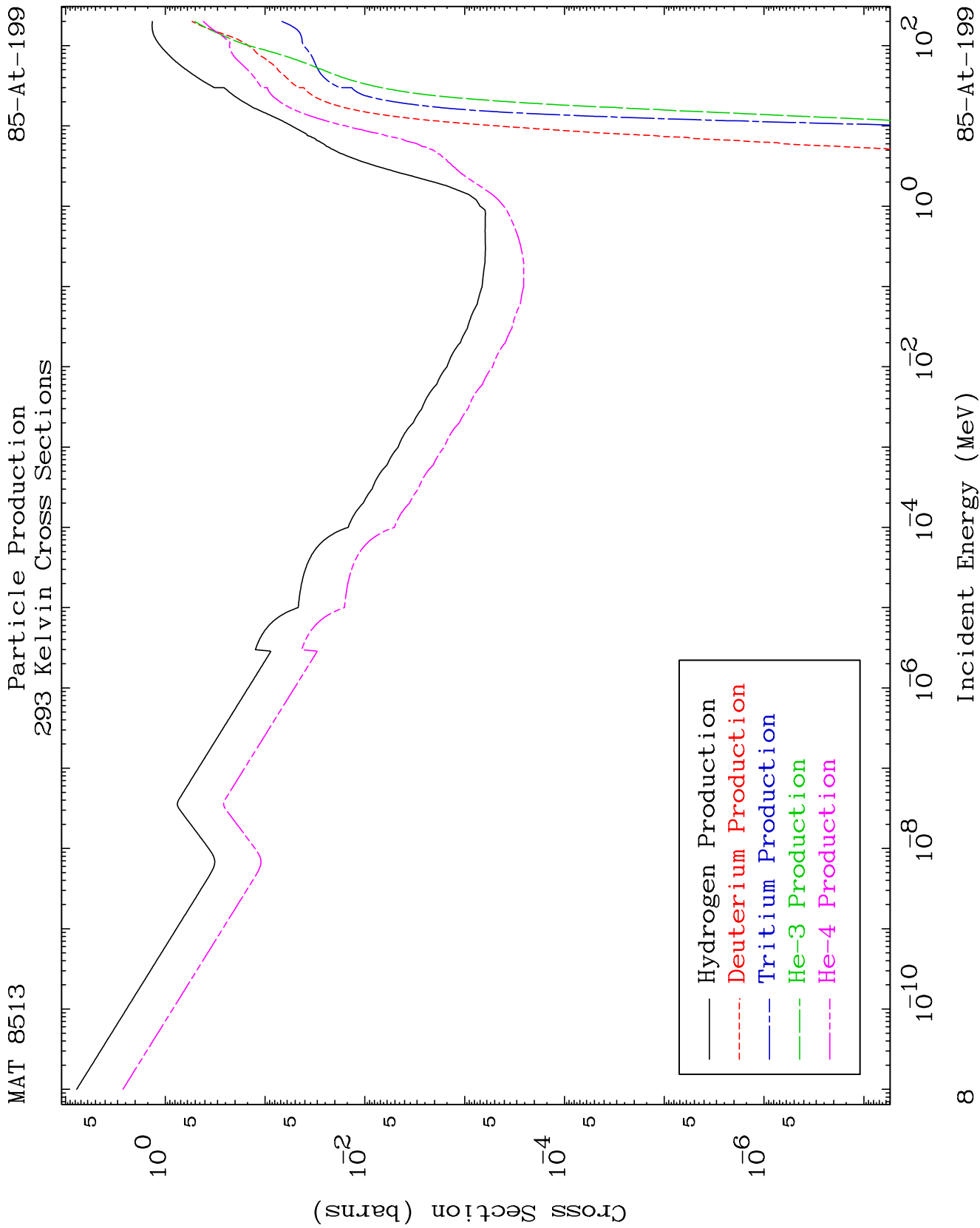


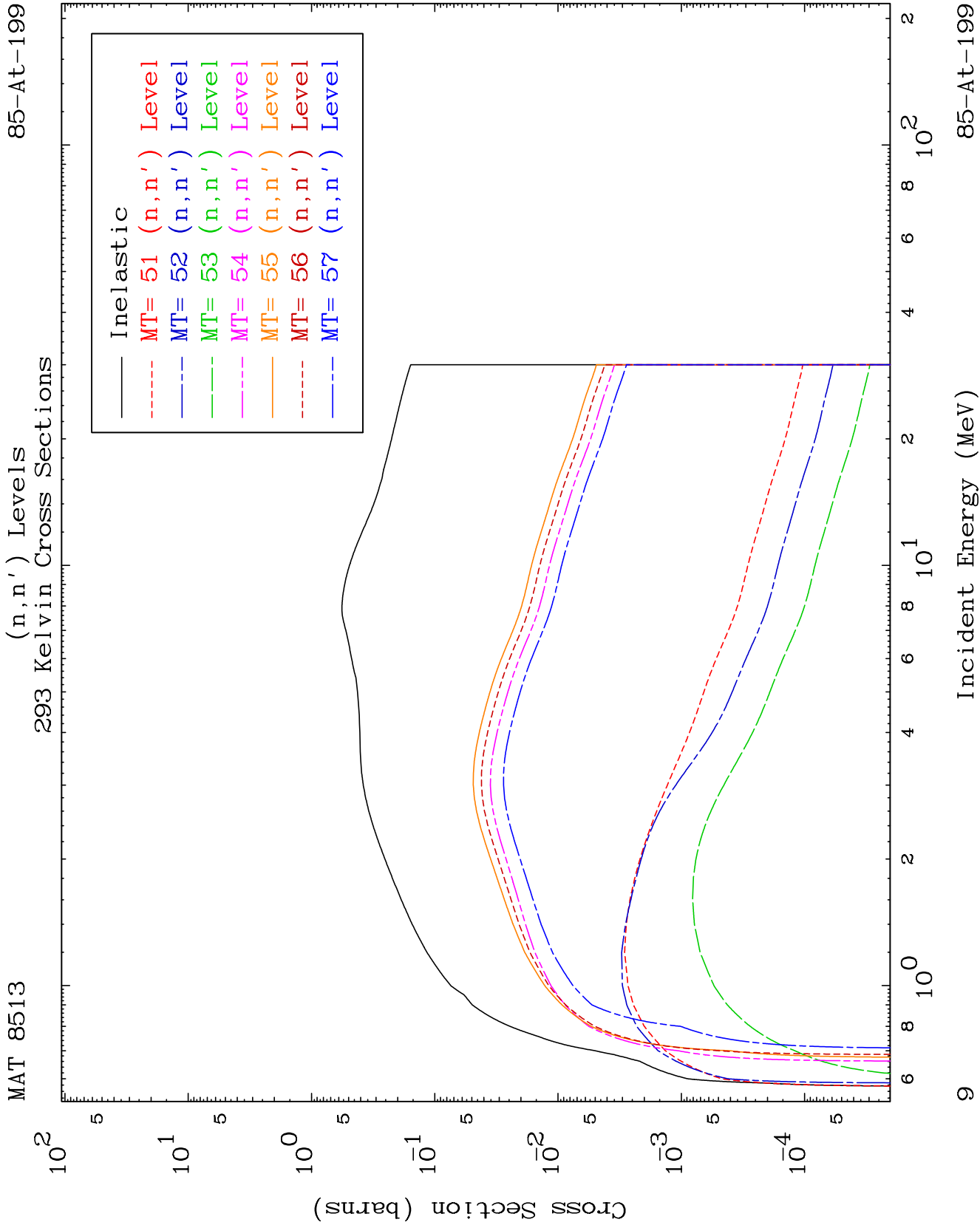










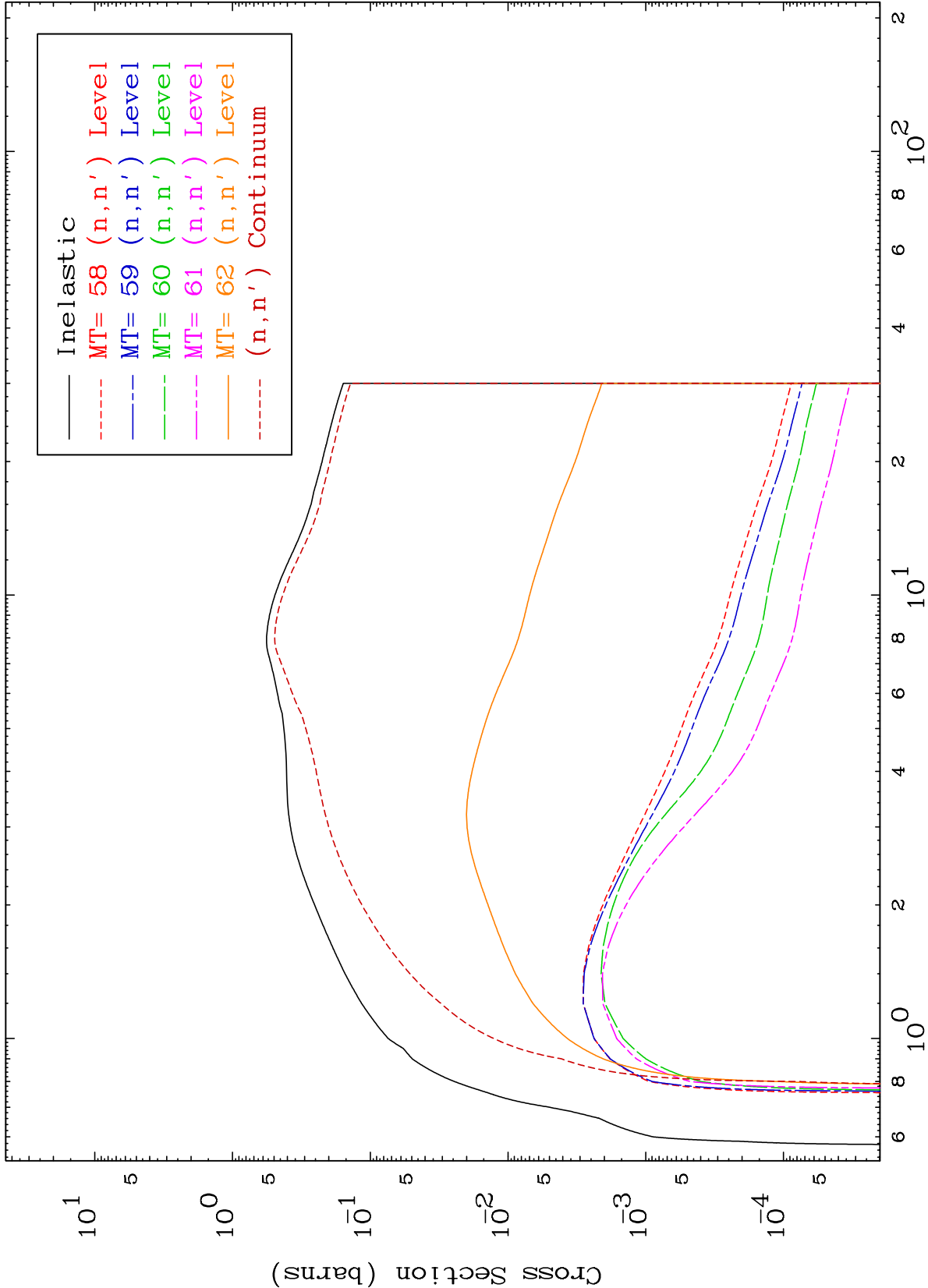


MAT 8513

(n,n') Levels

293 Kelvin Cross Sections

85-At-199



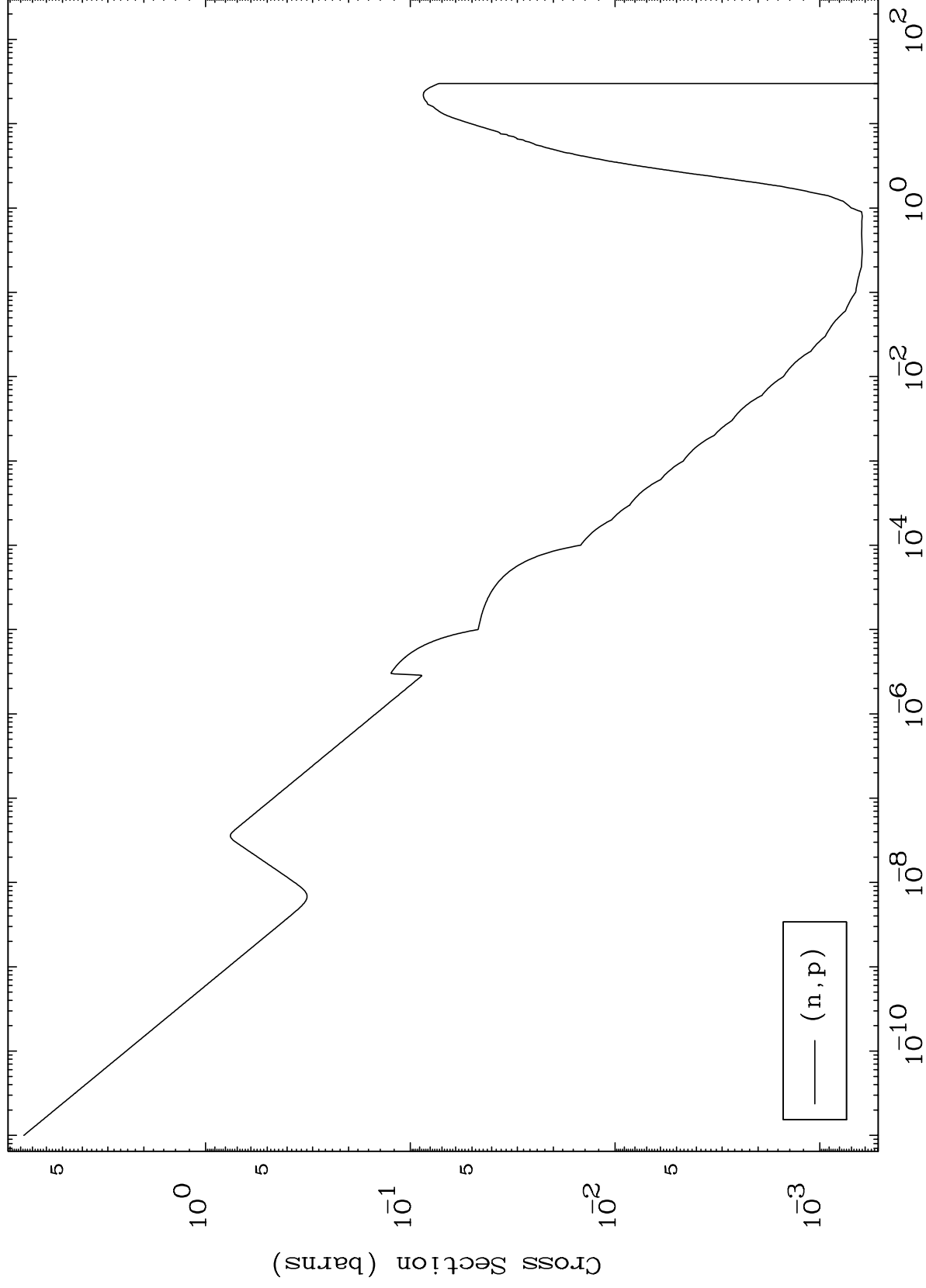
Incident Energy (MeV)

85-At-199

MAT 8513

(n,p) Levels
293 Kelvin Cross Sections

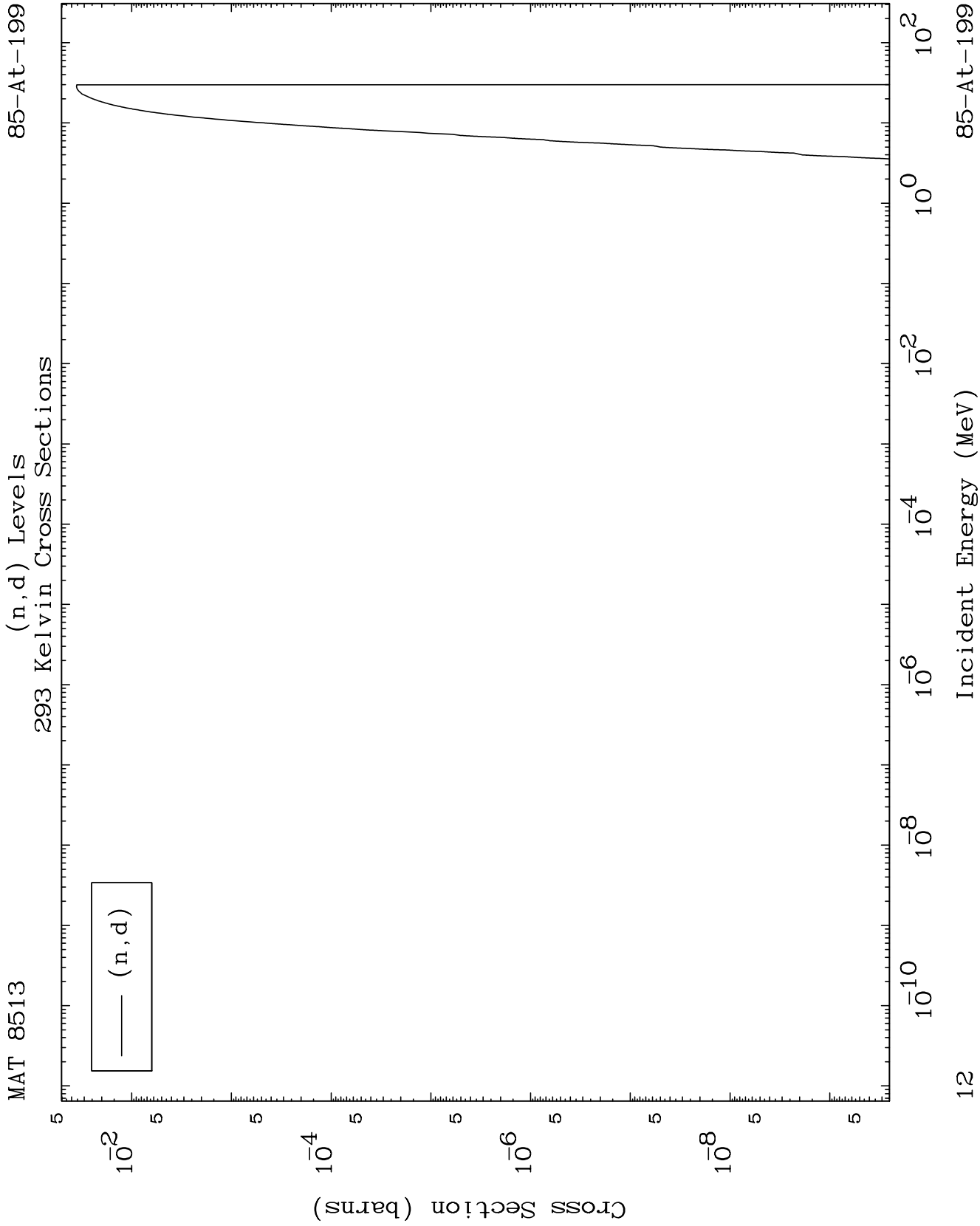
85-At-199

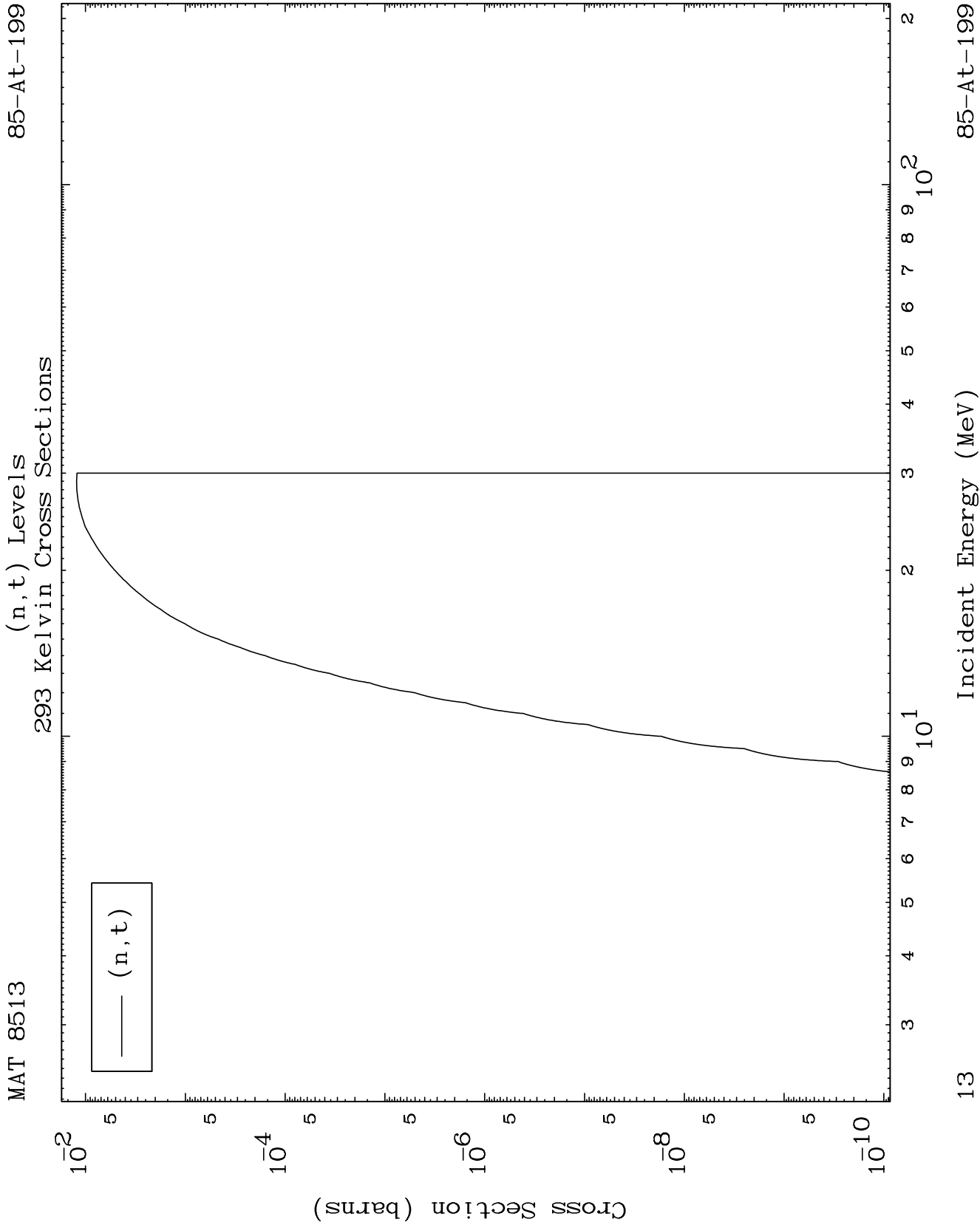


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Incident Energy (MeV)

85-At-199

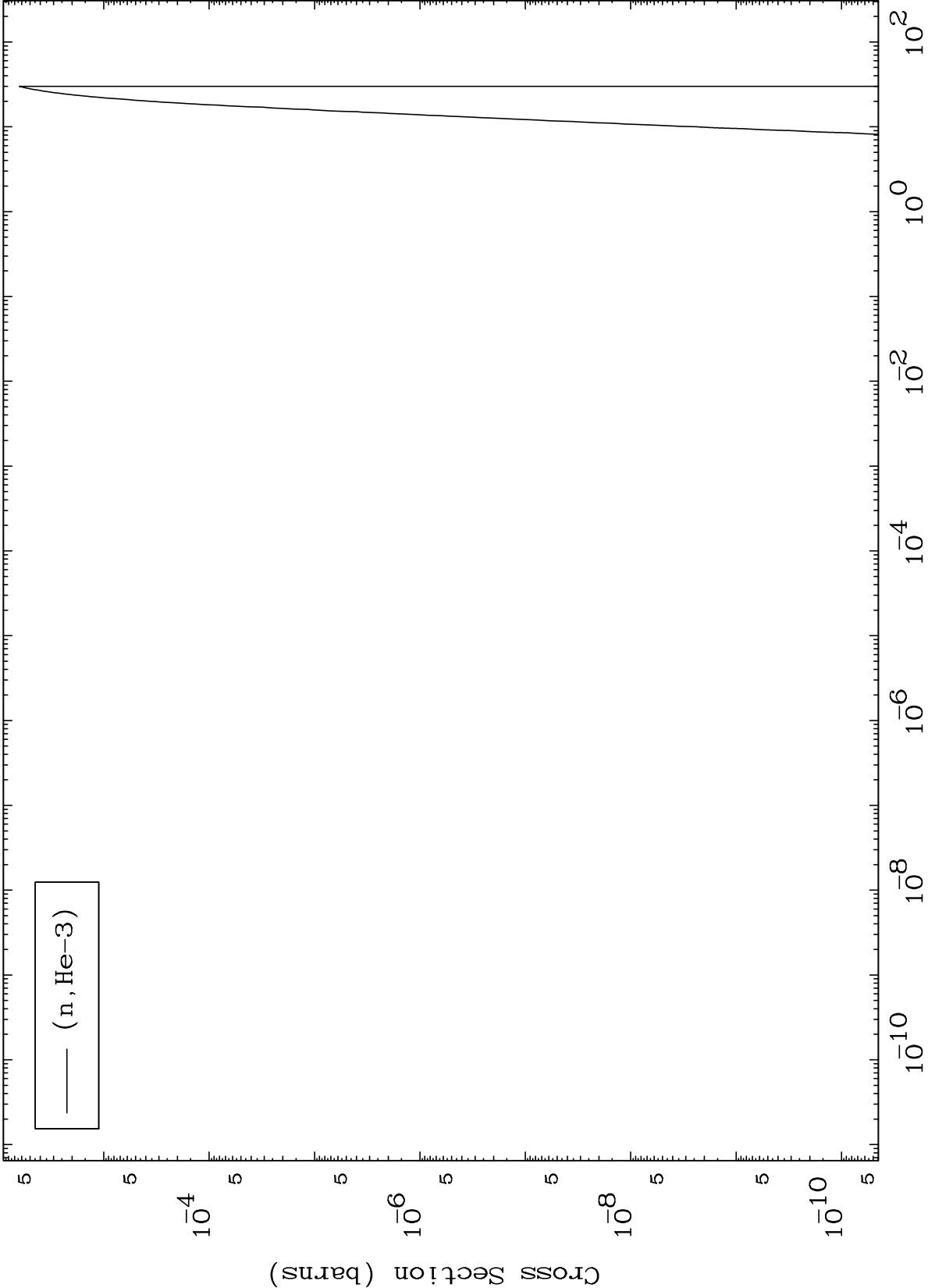




MAT 8513

(n,He3) Levels
293 Kelvin Cross Sections

85-At-199



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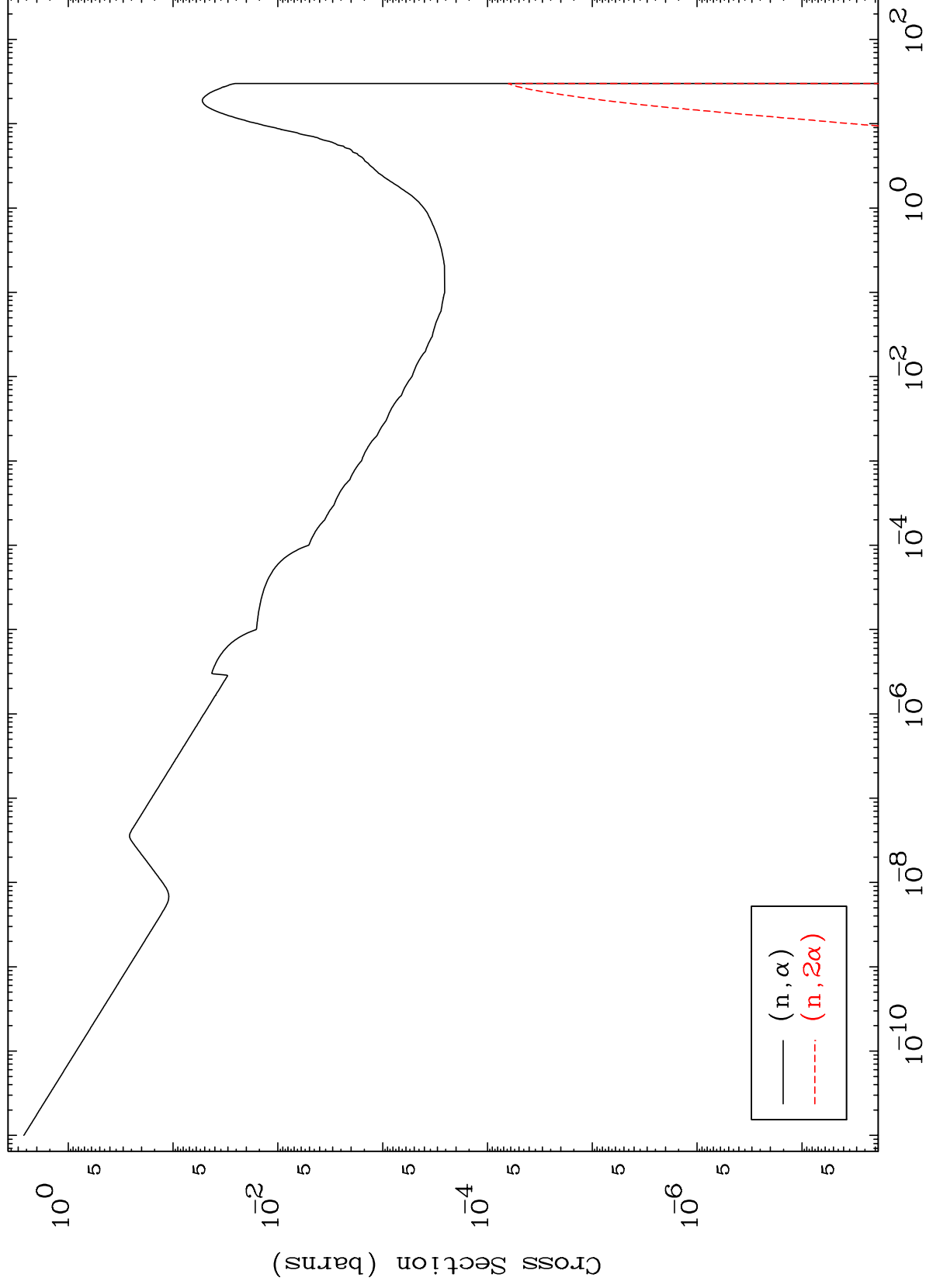
Incident Energy (MeV)

85-At-199

MAT 8513

(n, α) Levels
293 Kelvin Cross Sections

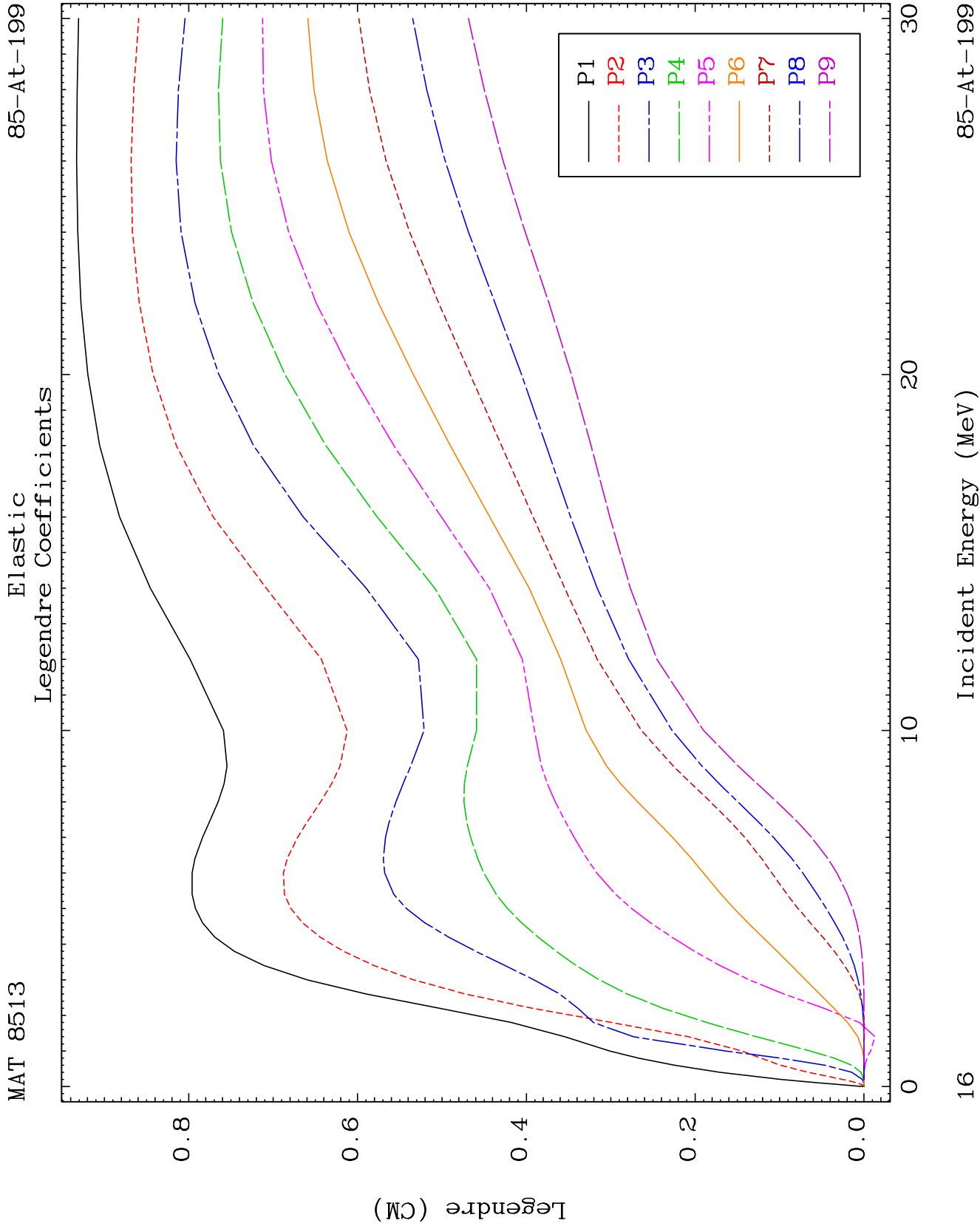
85-At-199

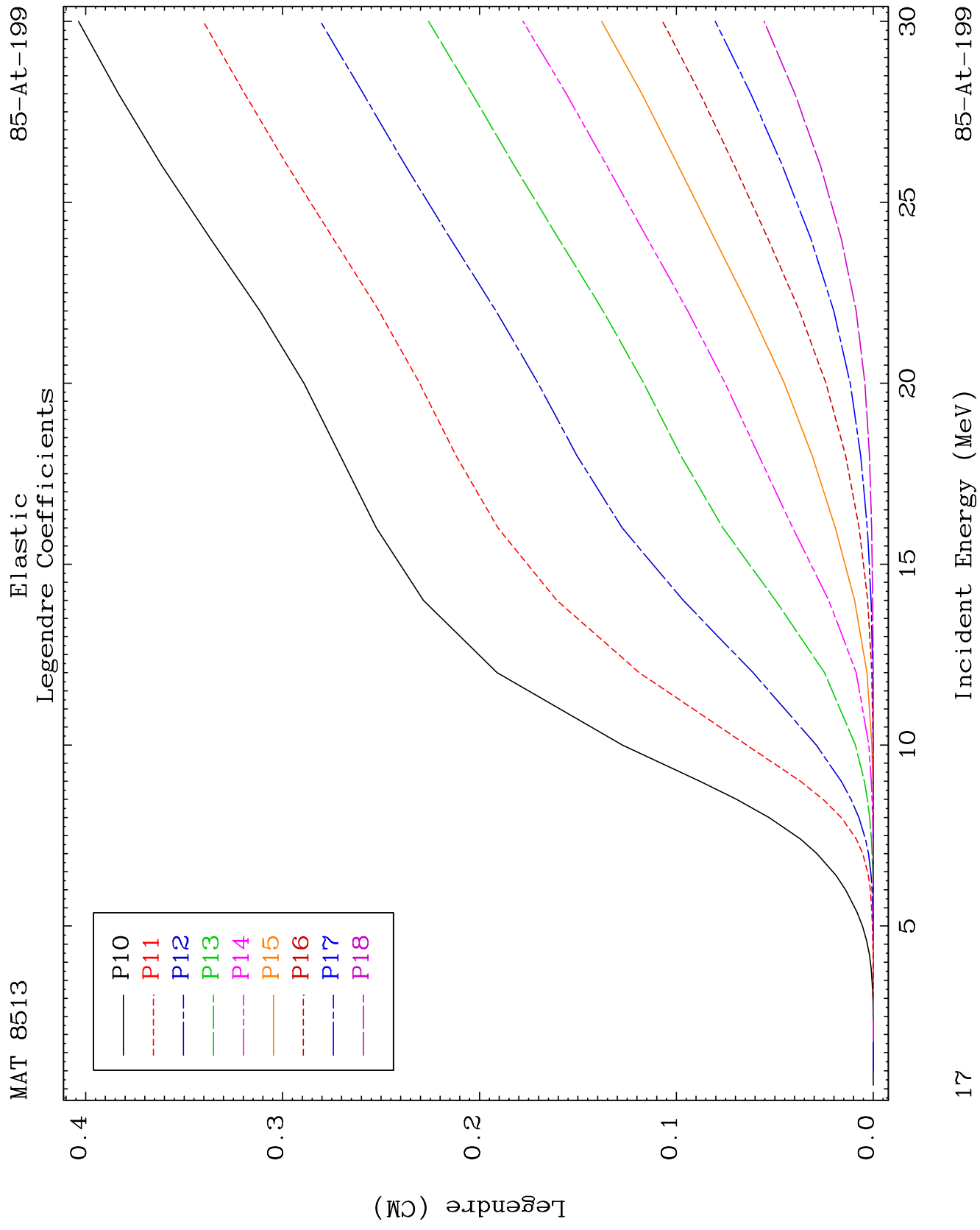


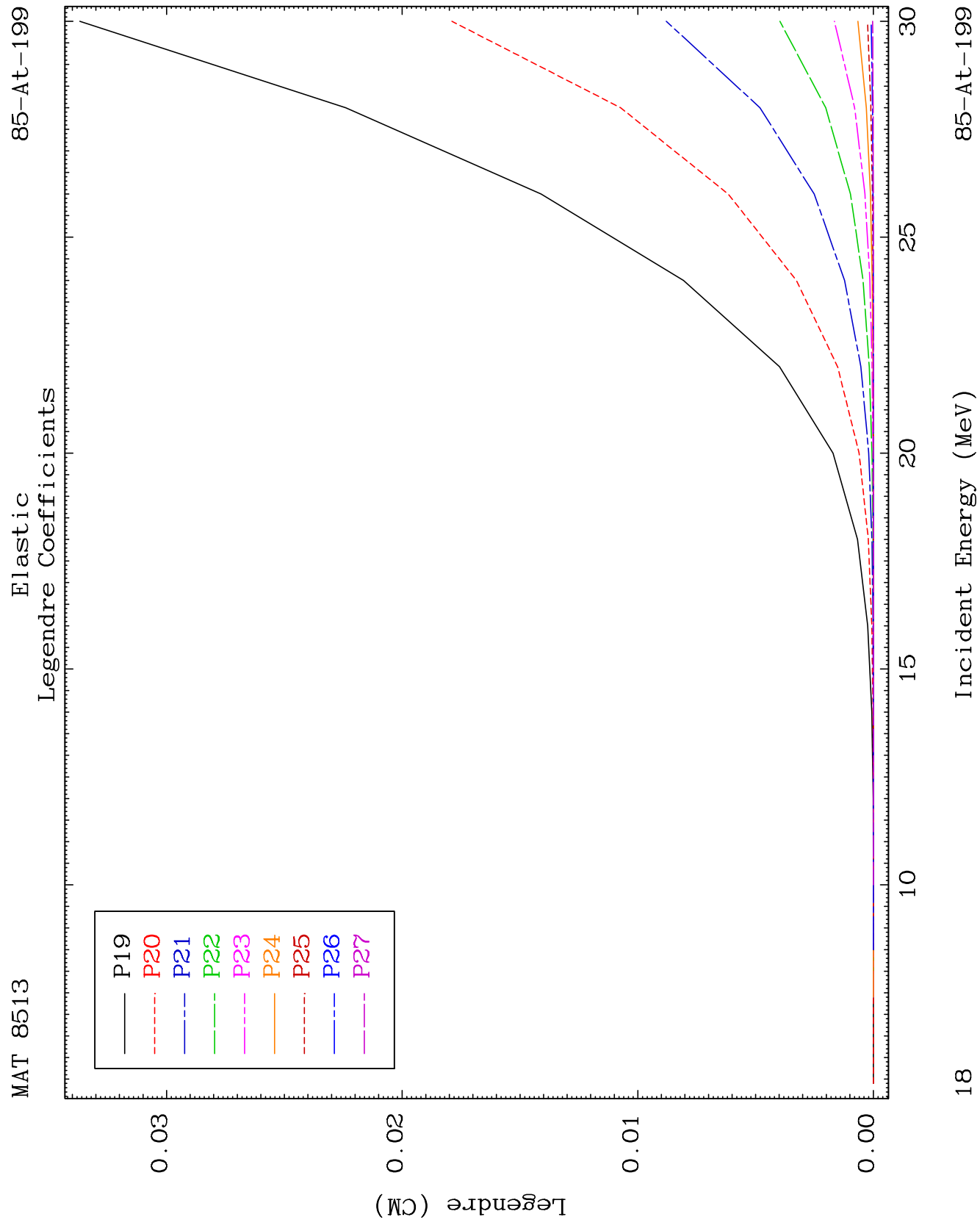
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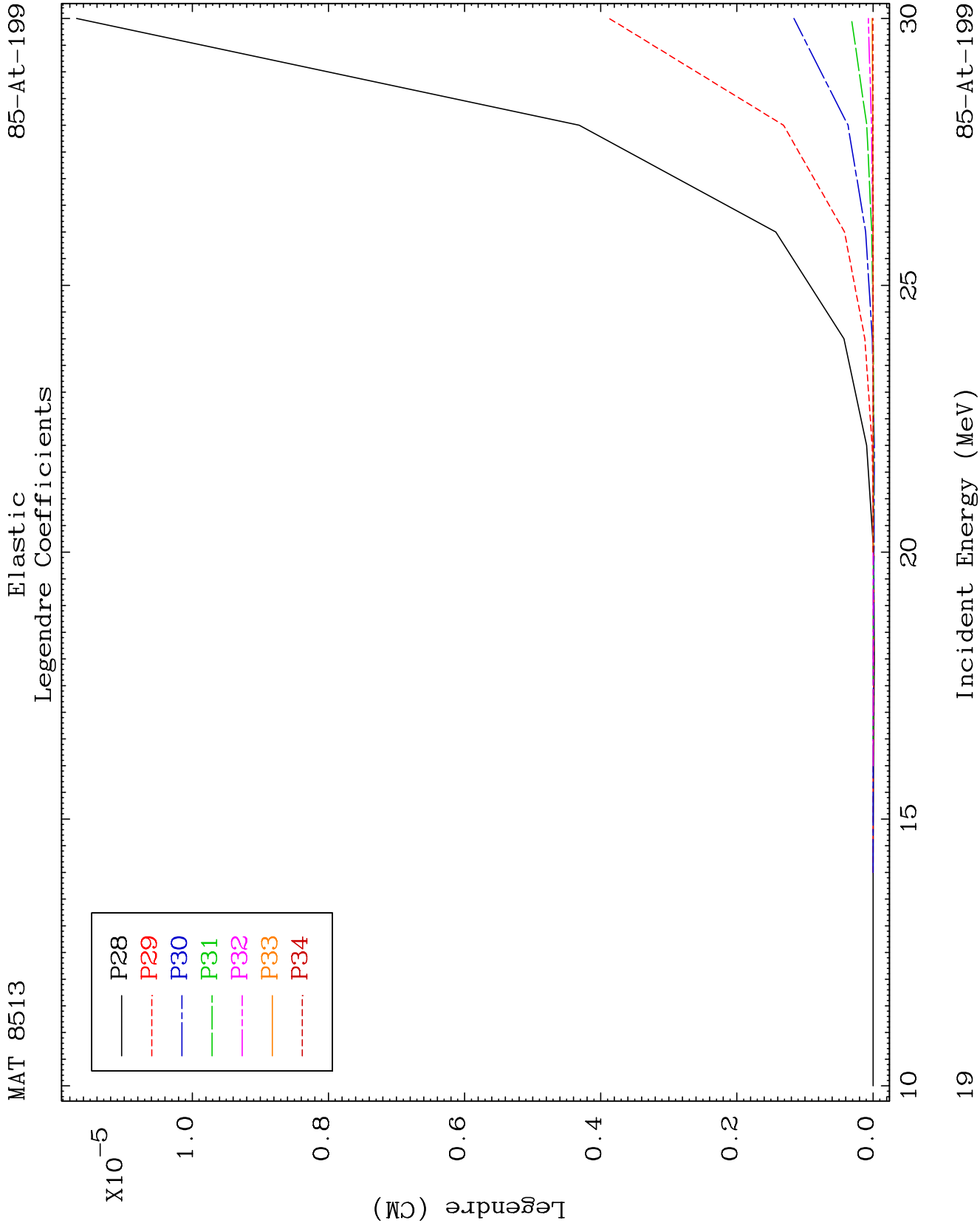
Incident Energy (MeV)

85-At-199





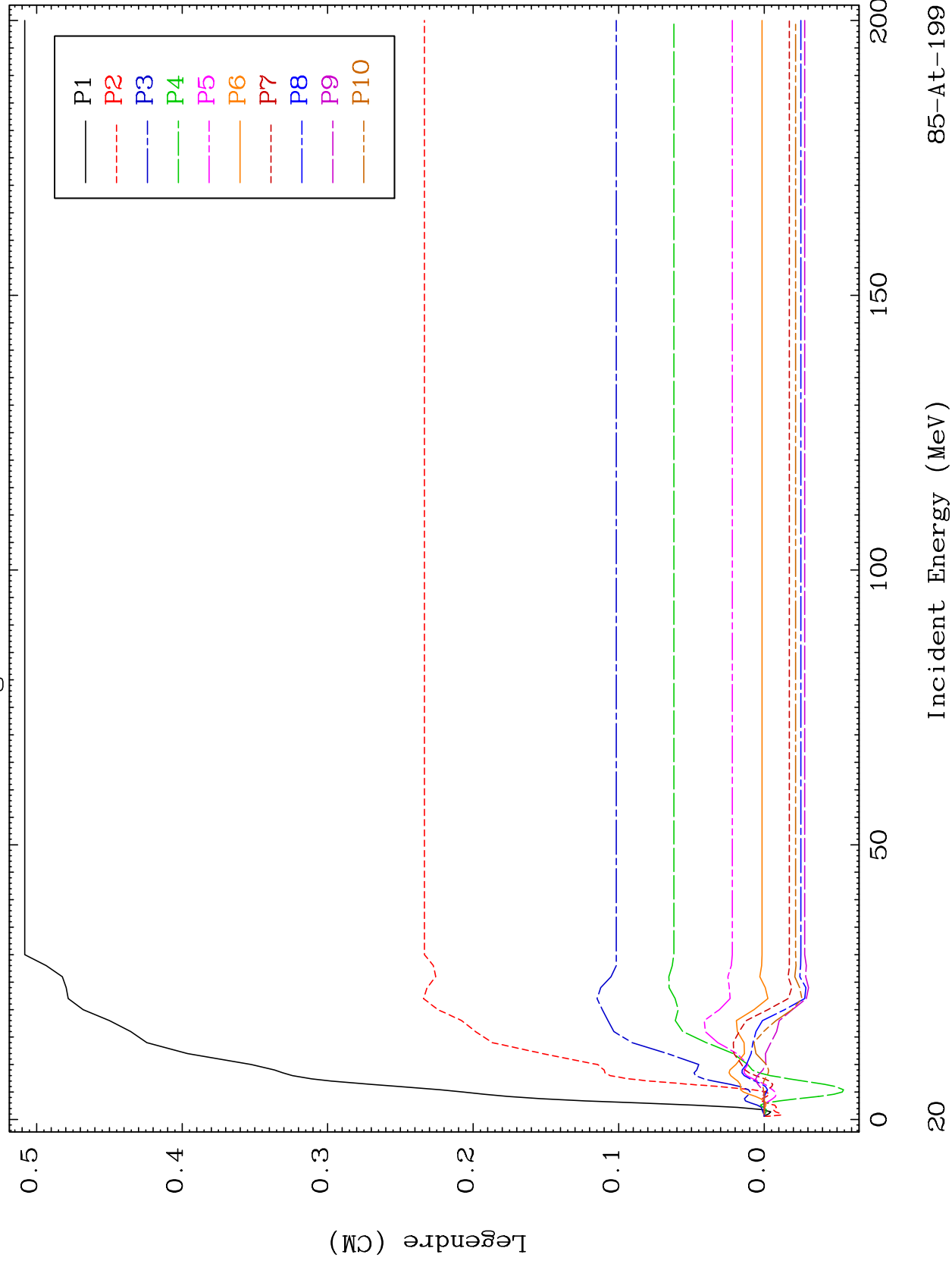


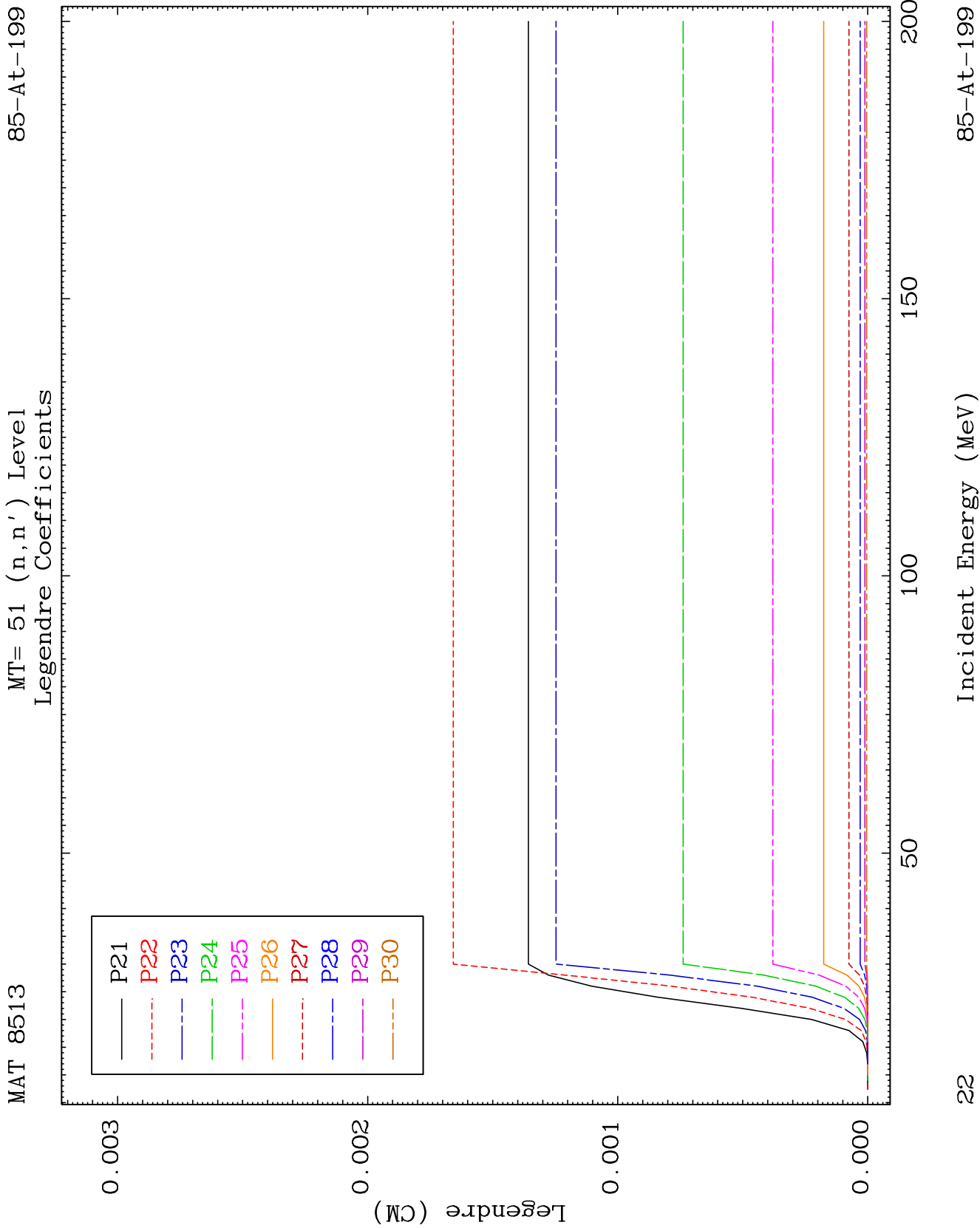


MAT 8513

MT= 51 (n, n') Level
Legendre Coefficients

85-At-199

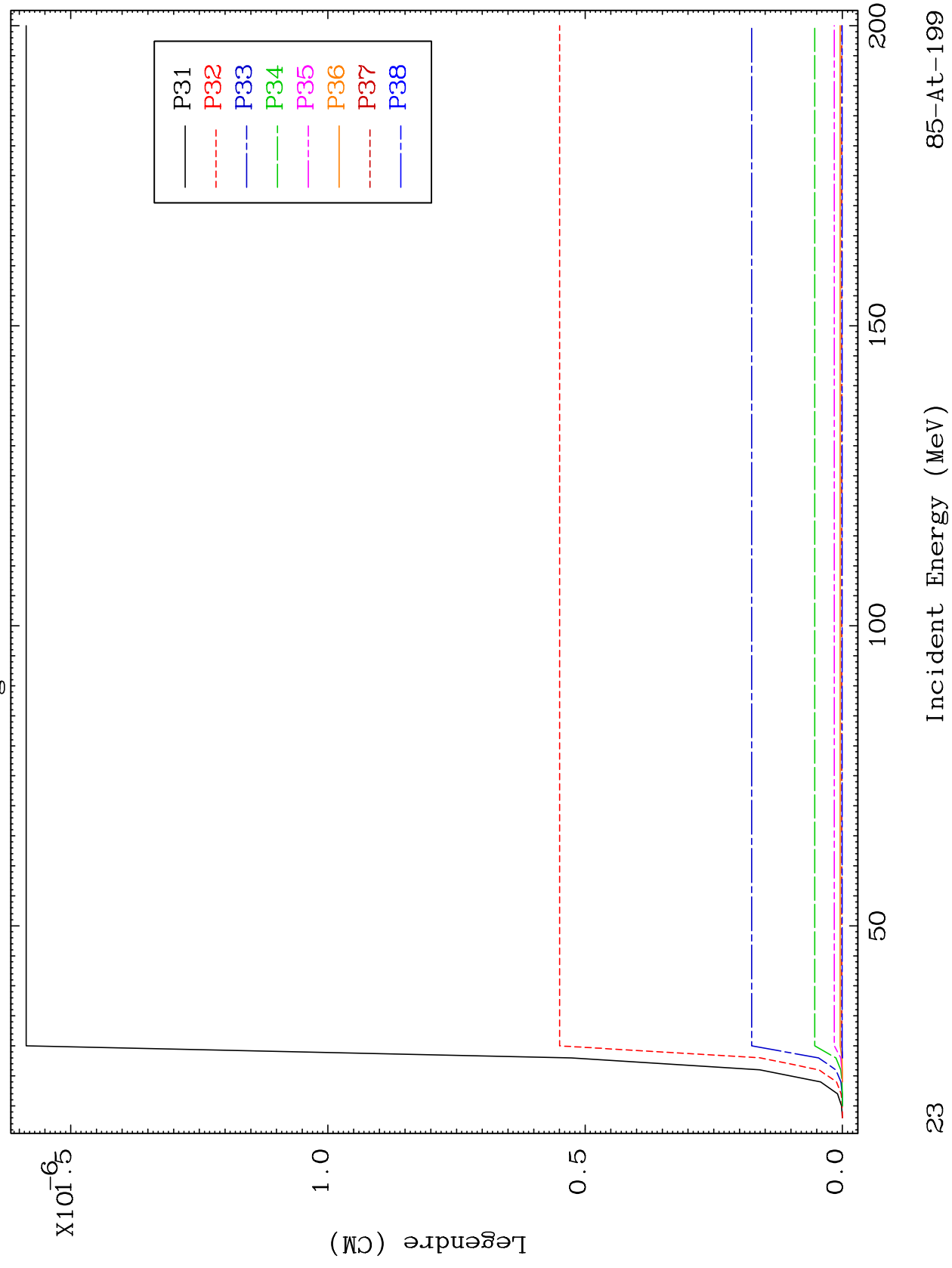


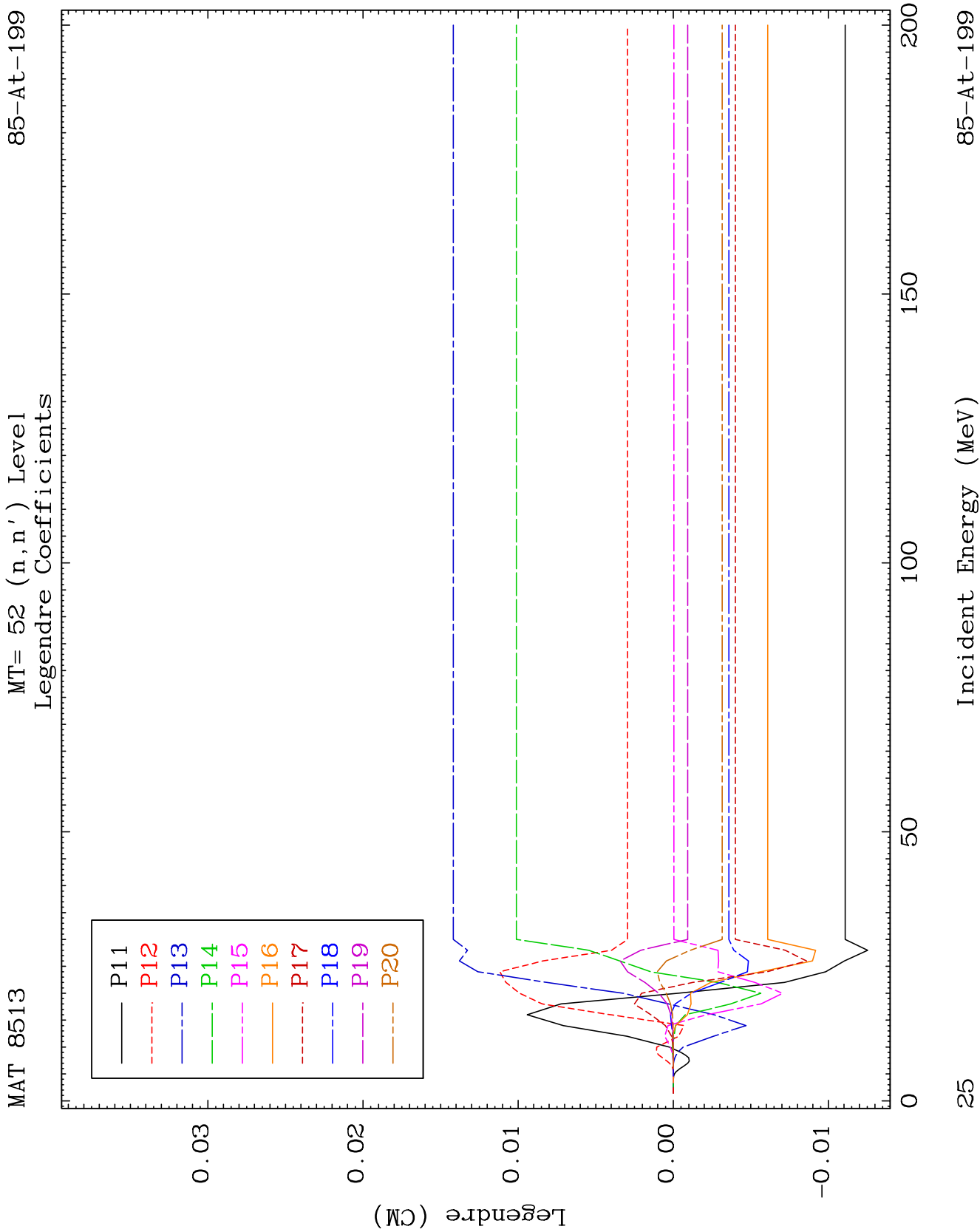


MAT 8513

MT= 51 (n, n') Level
Legendre Coefficients

85-At-199

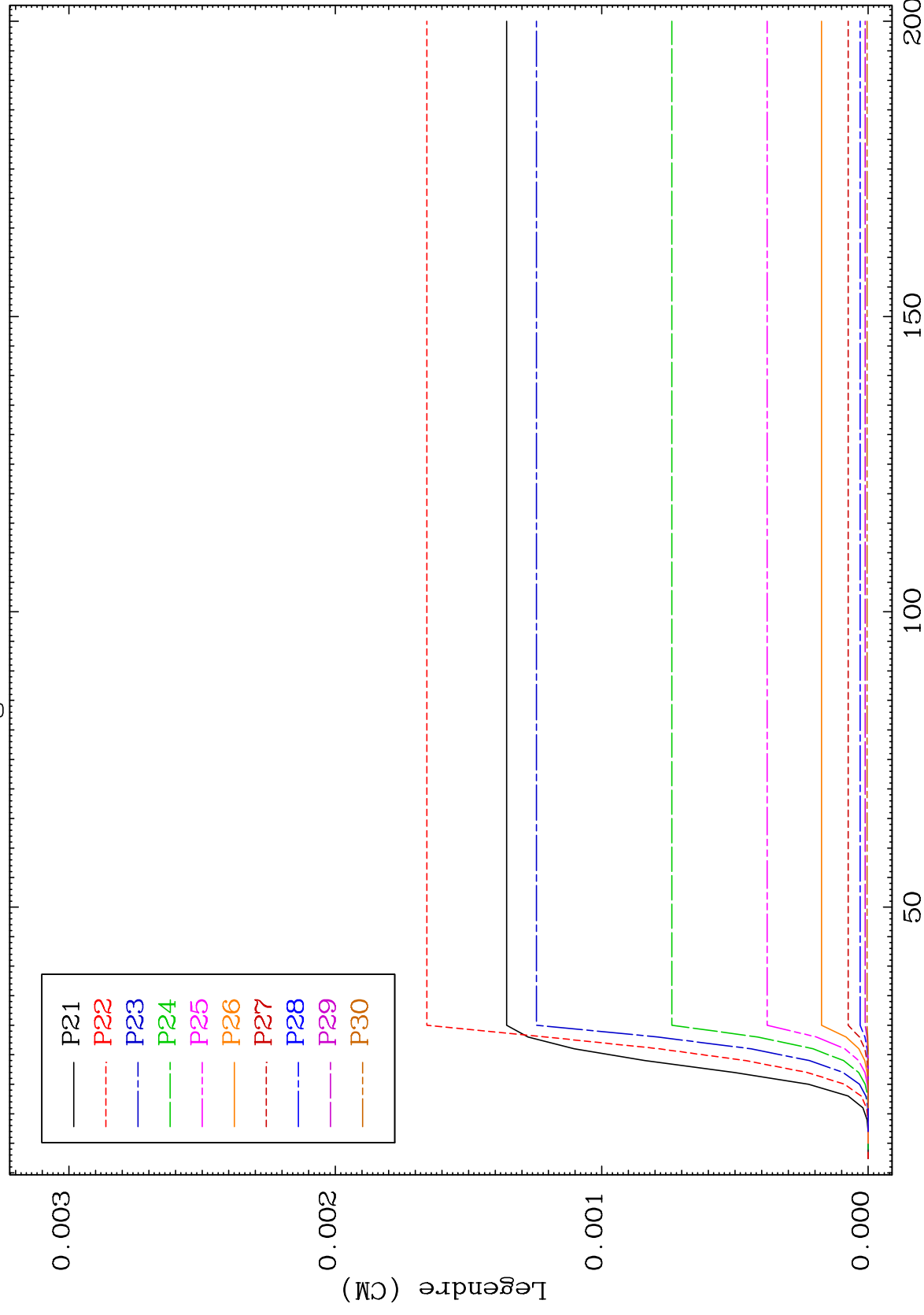




MAT 8513

MT= 52 (n, n') Level
Legendre Coefficients

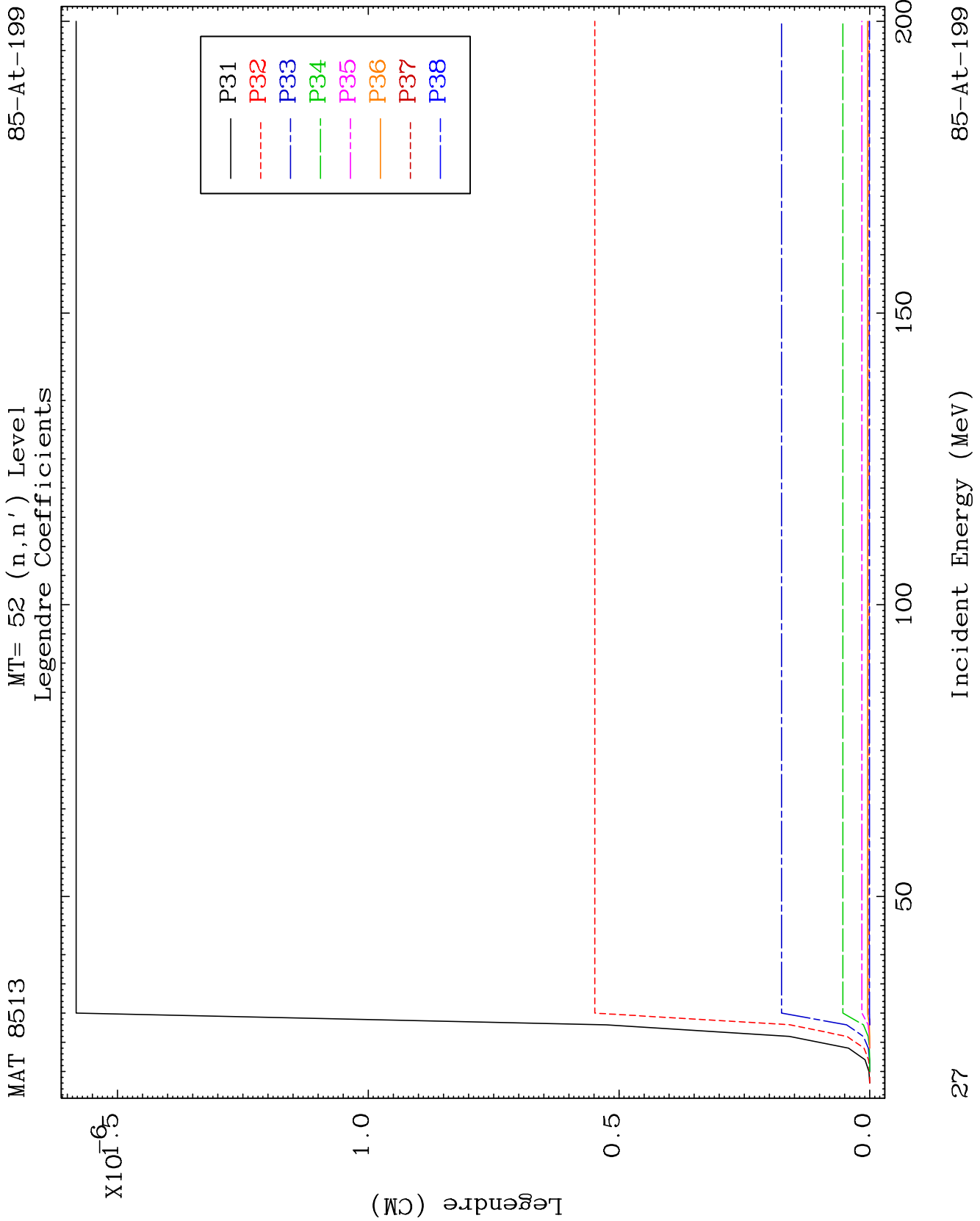
85-At-199

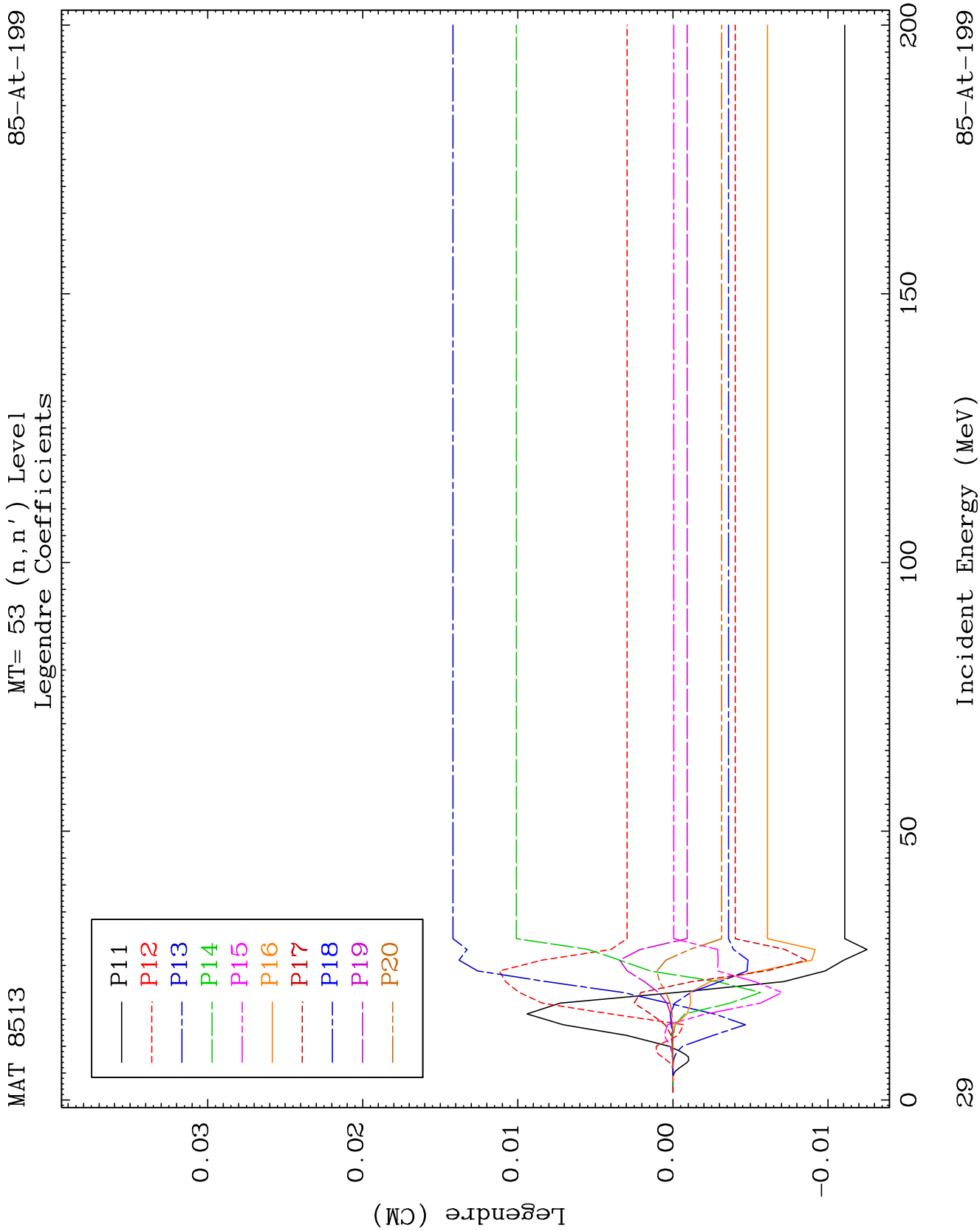


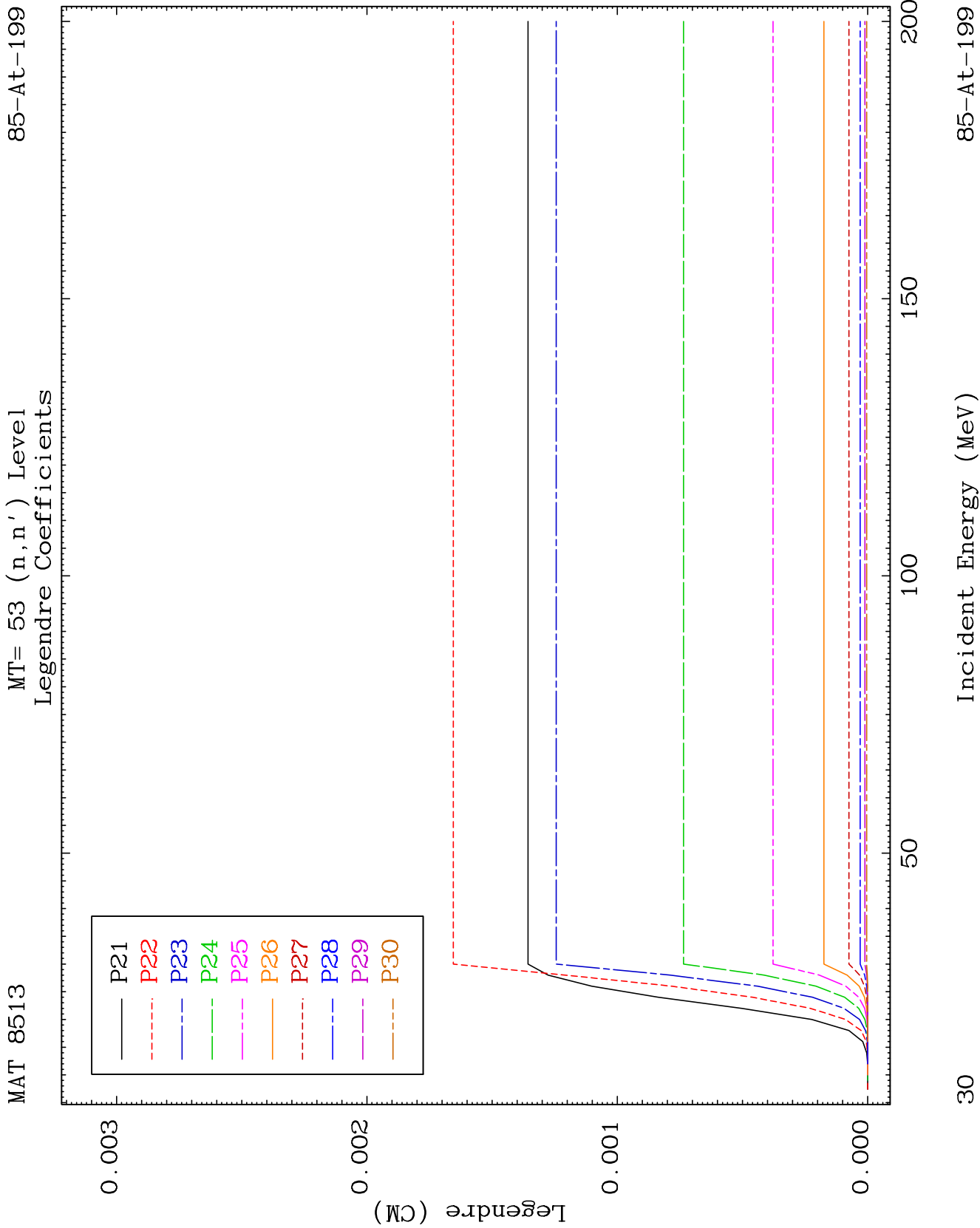
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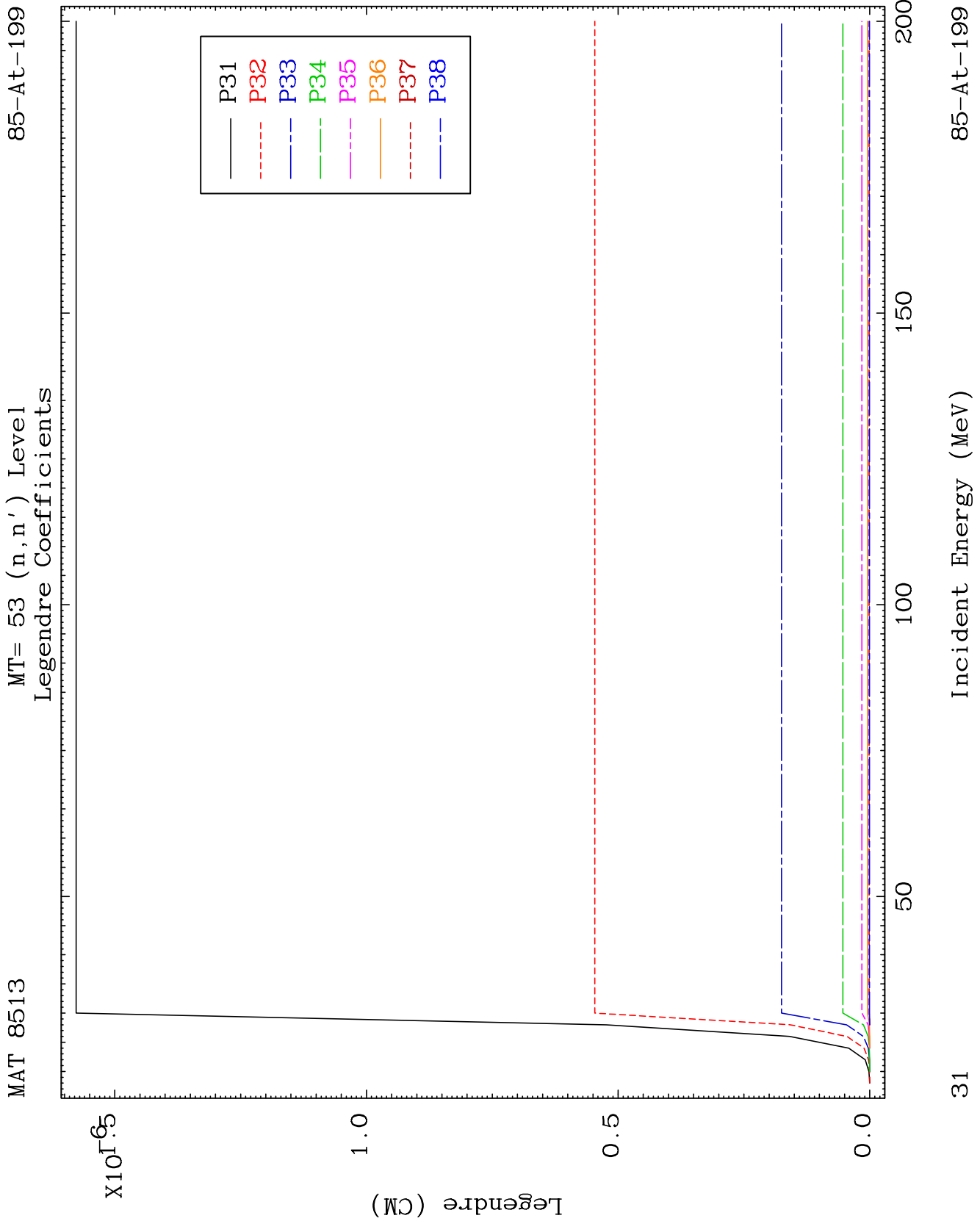
Incident Energy (MeV)

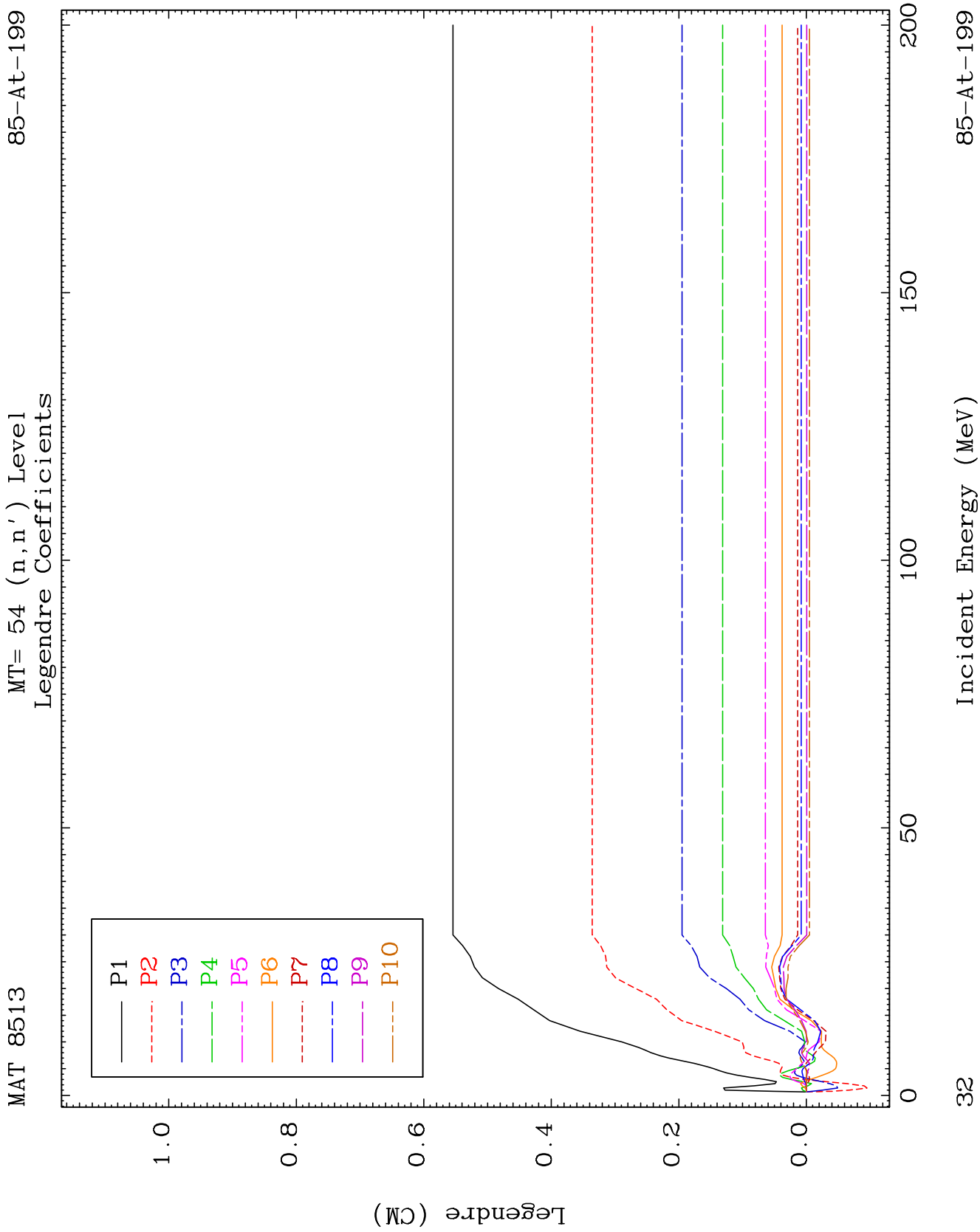
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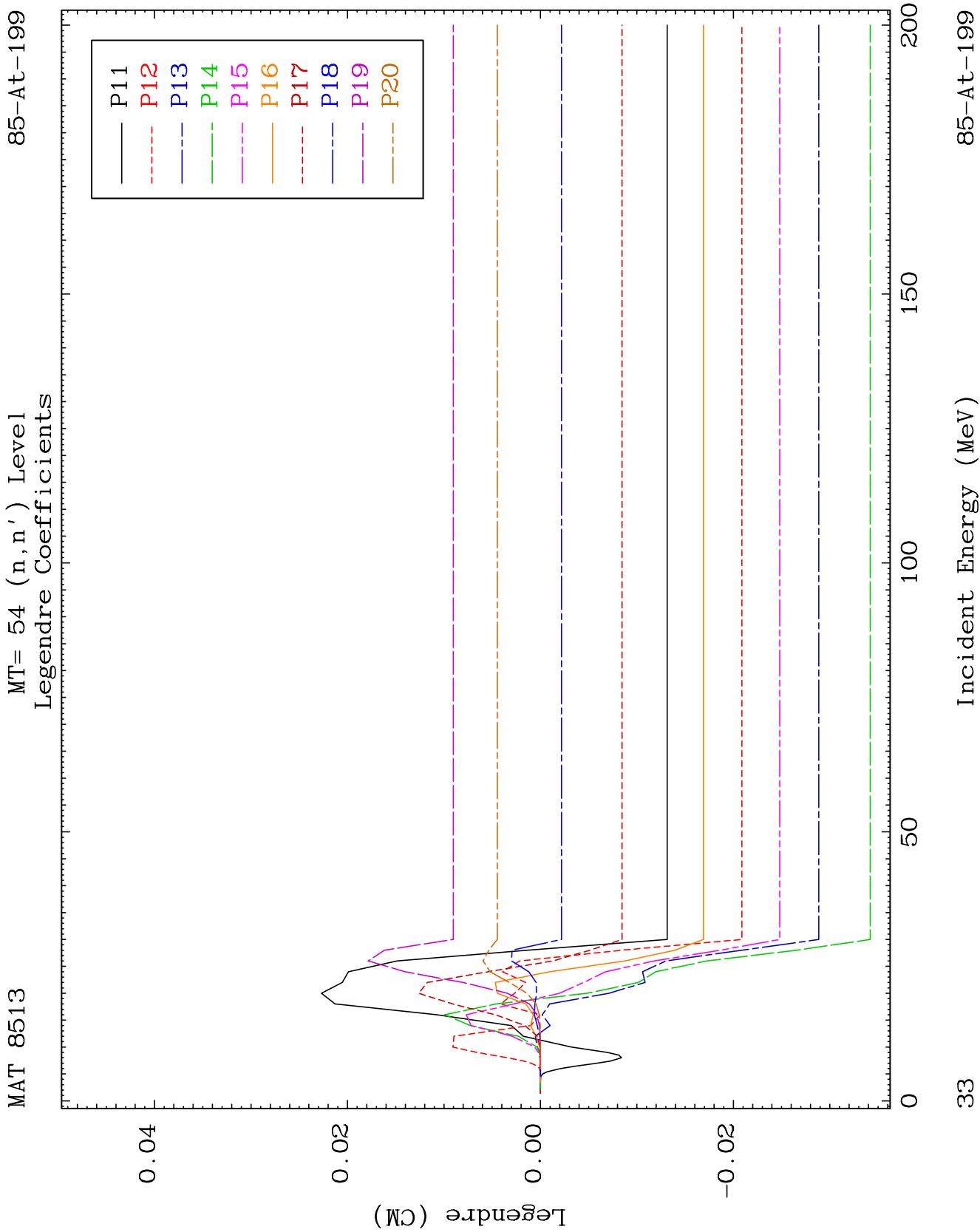


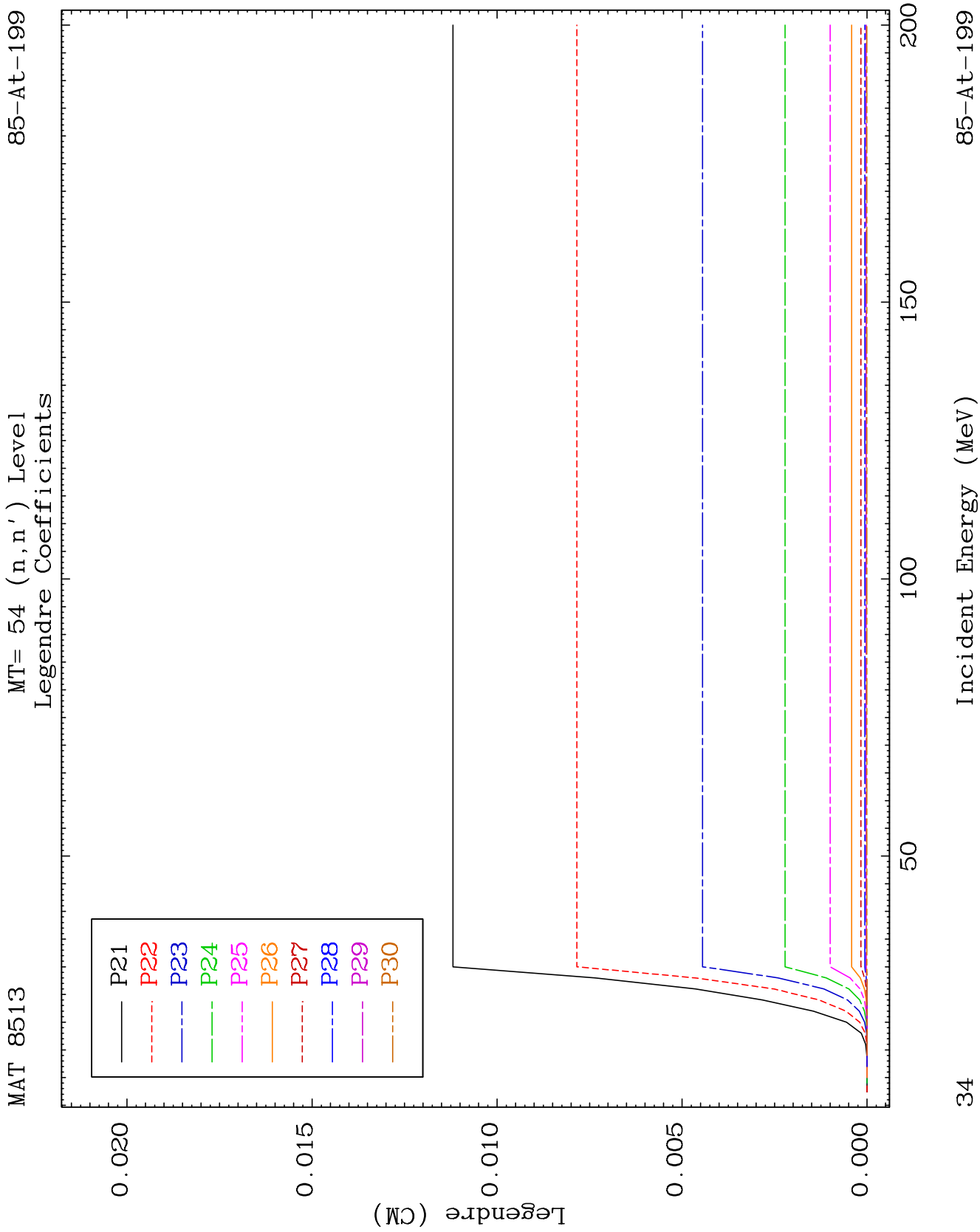


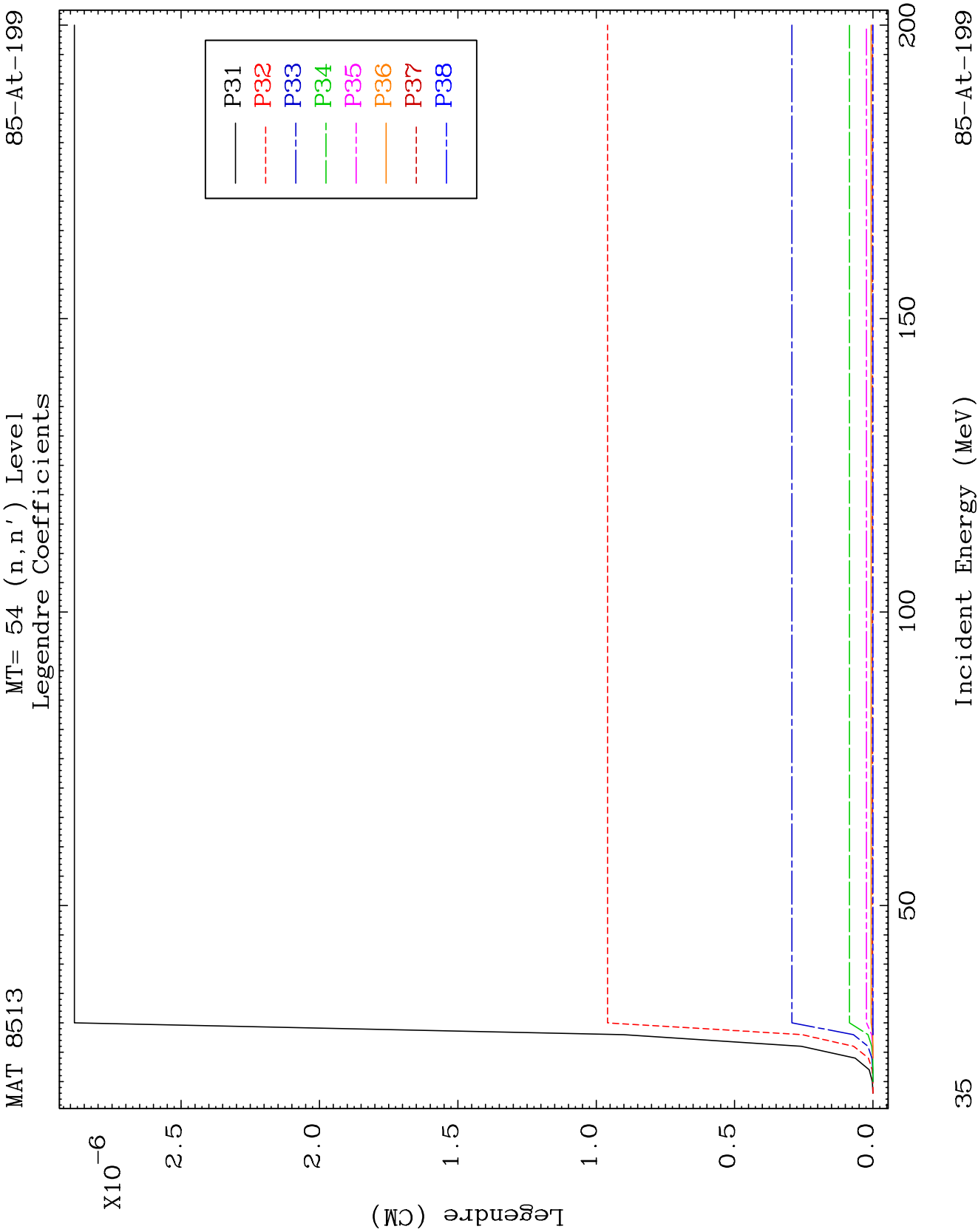








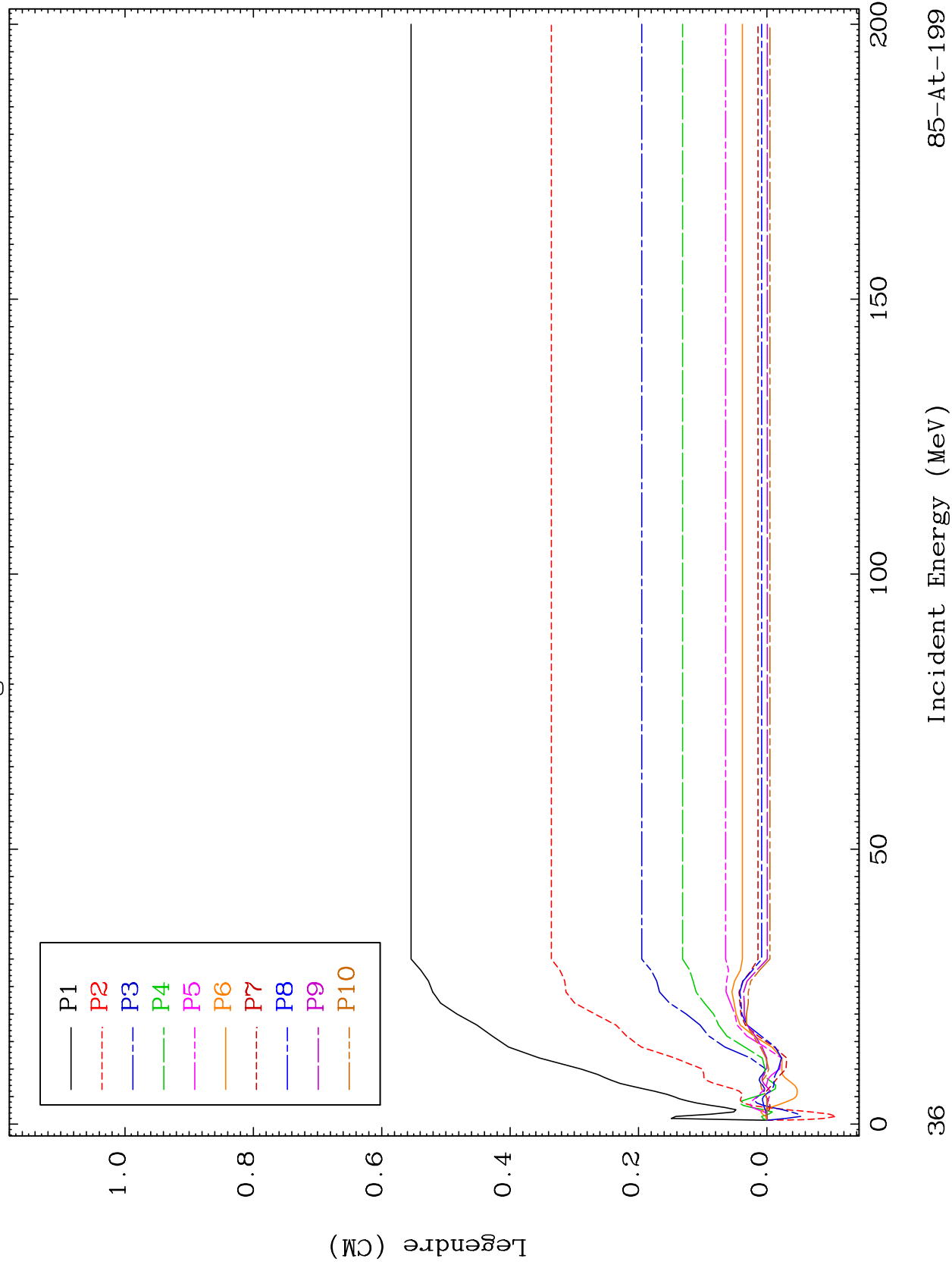


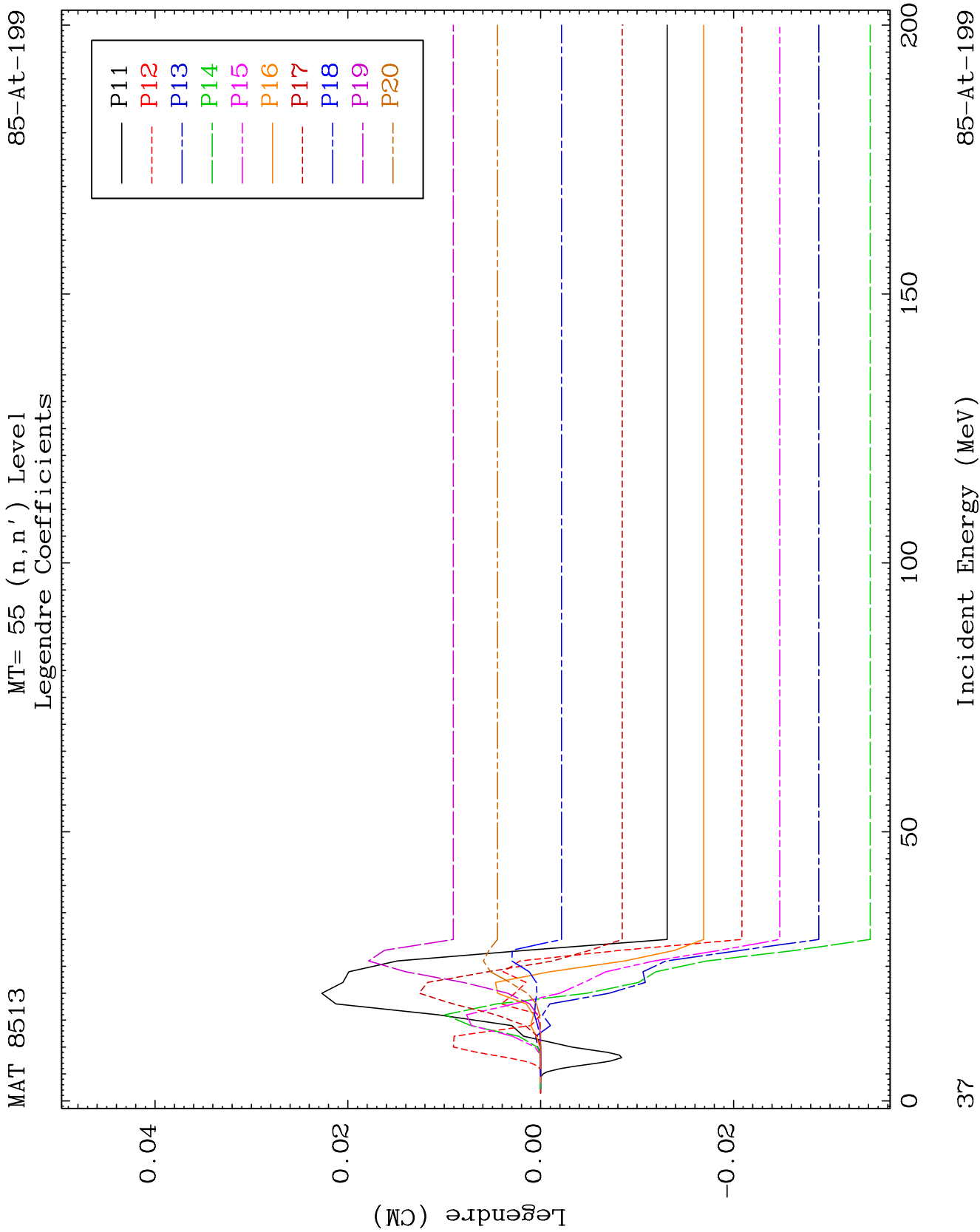


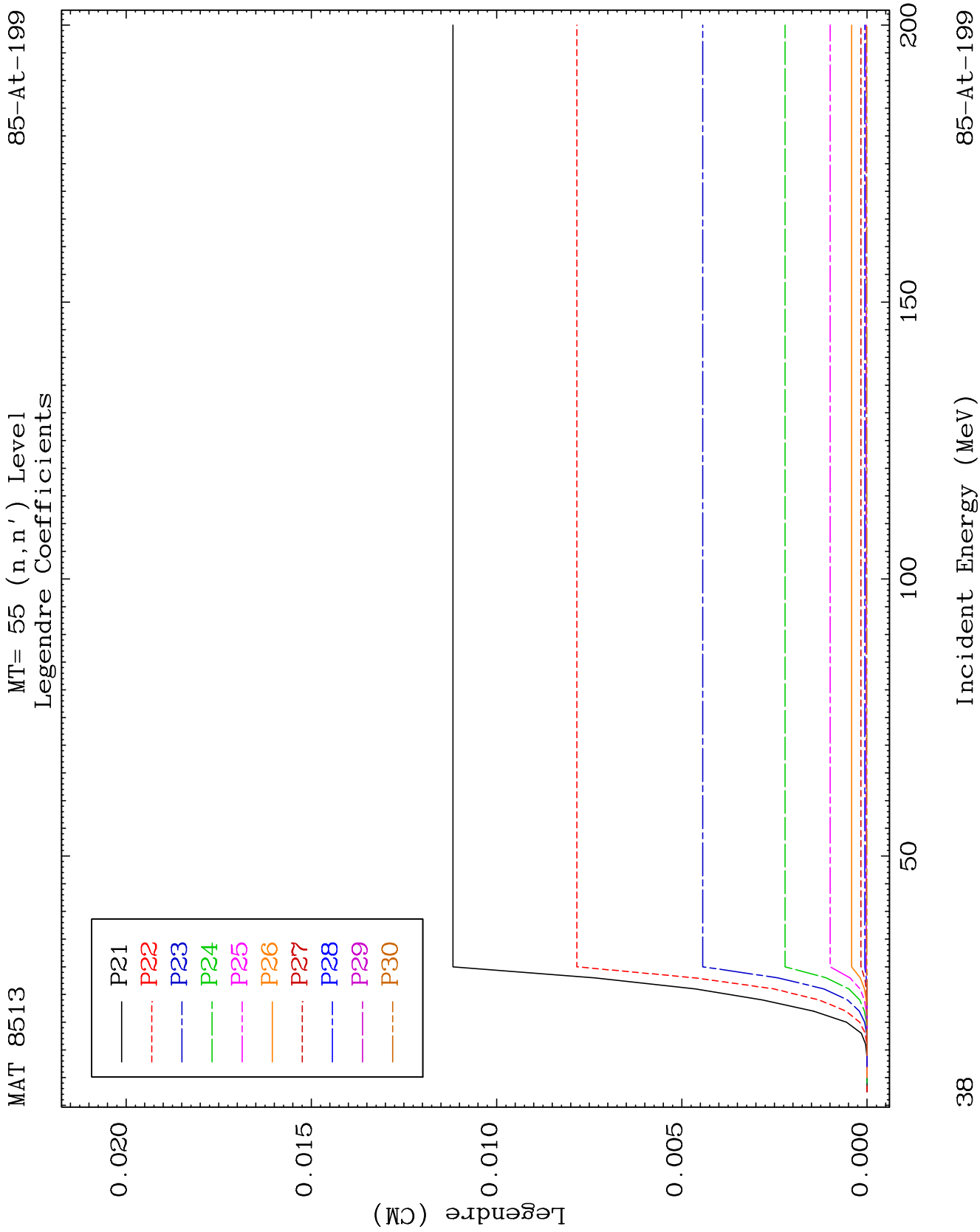
MAT 8513

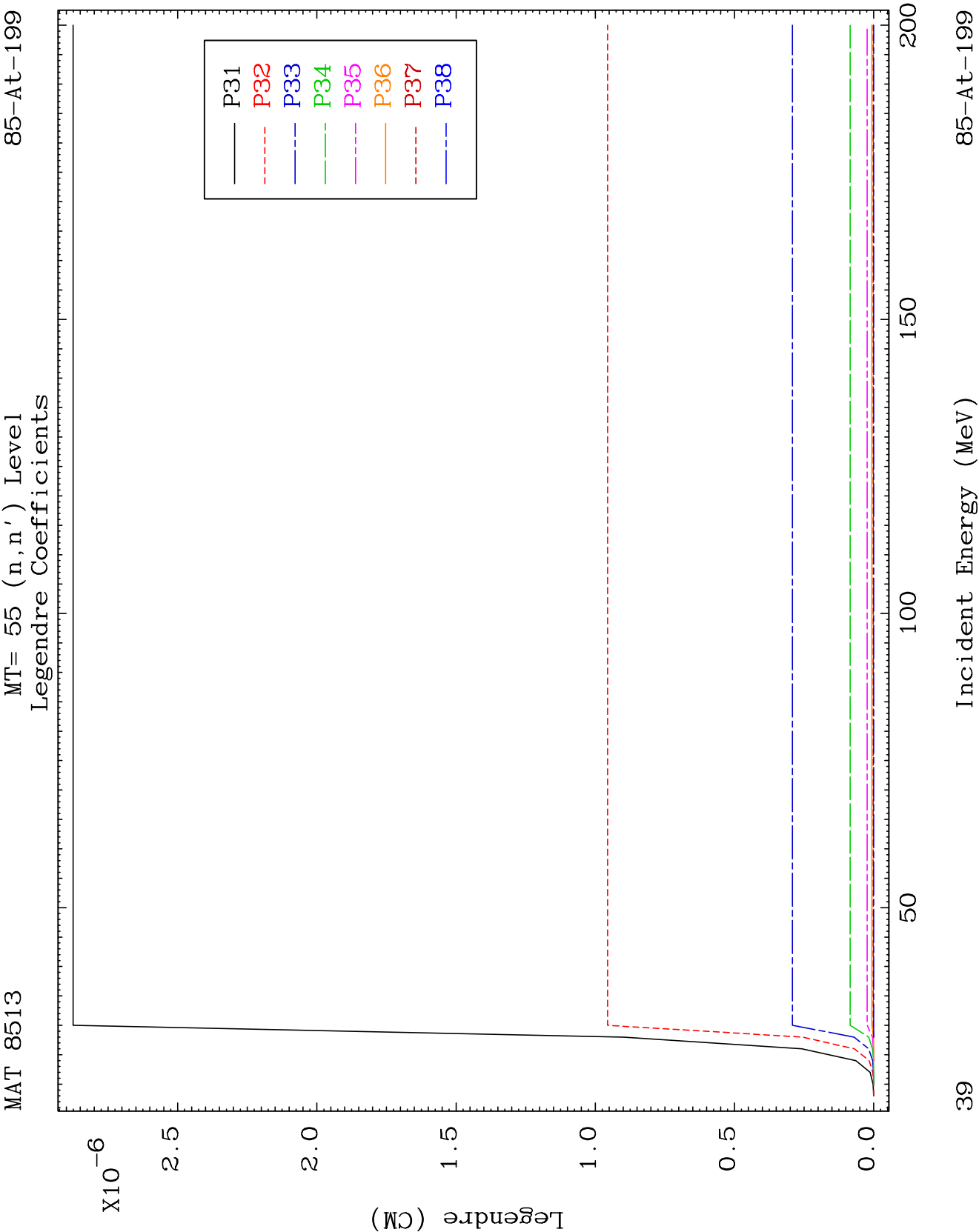
MT= 55 (n, n') Level
Legendre Coefficients

85-At-199





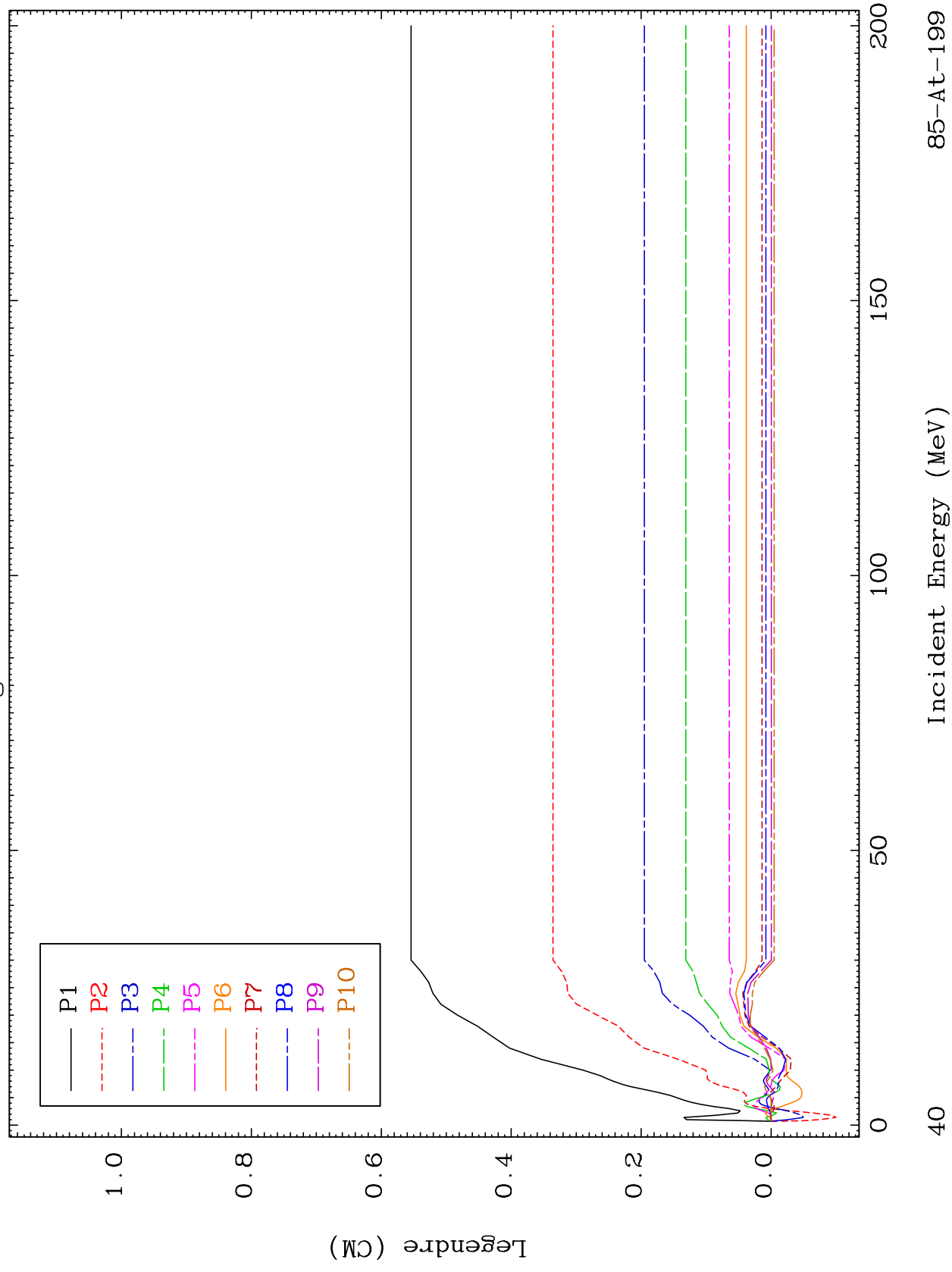


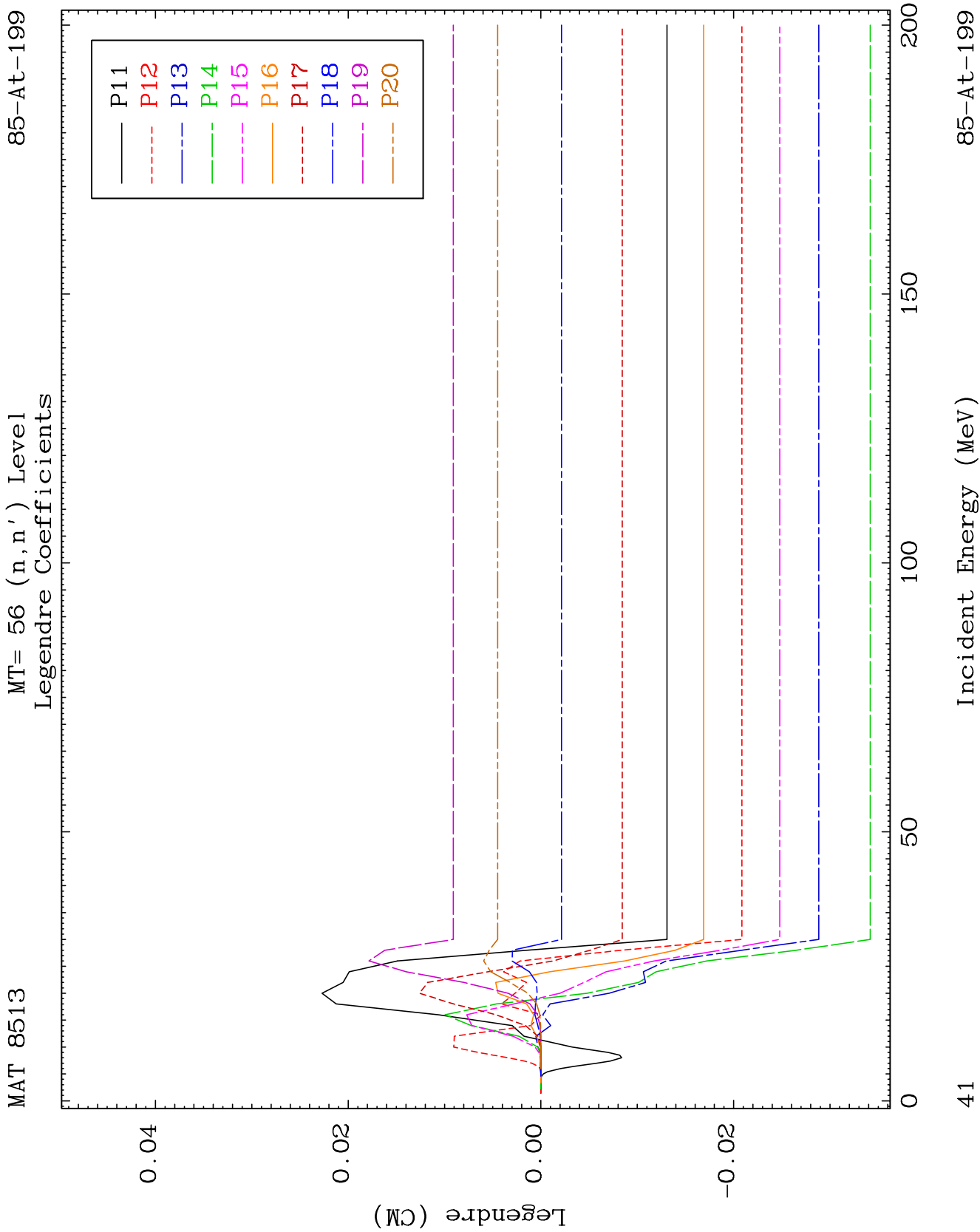


MAT 8513

MT= 56 (n, n') Level
Legendre Coefficients

85-At-199

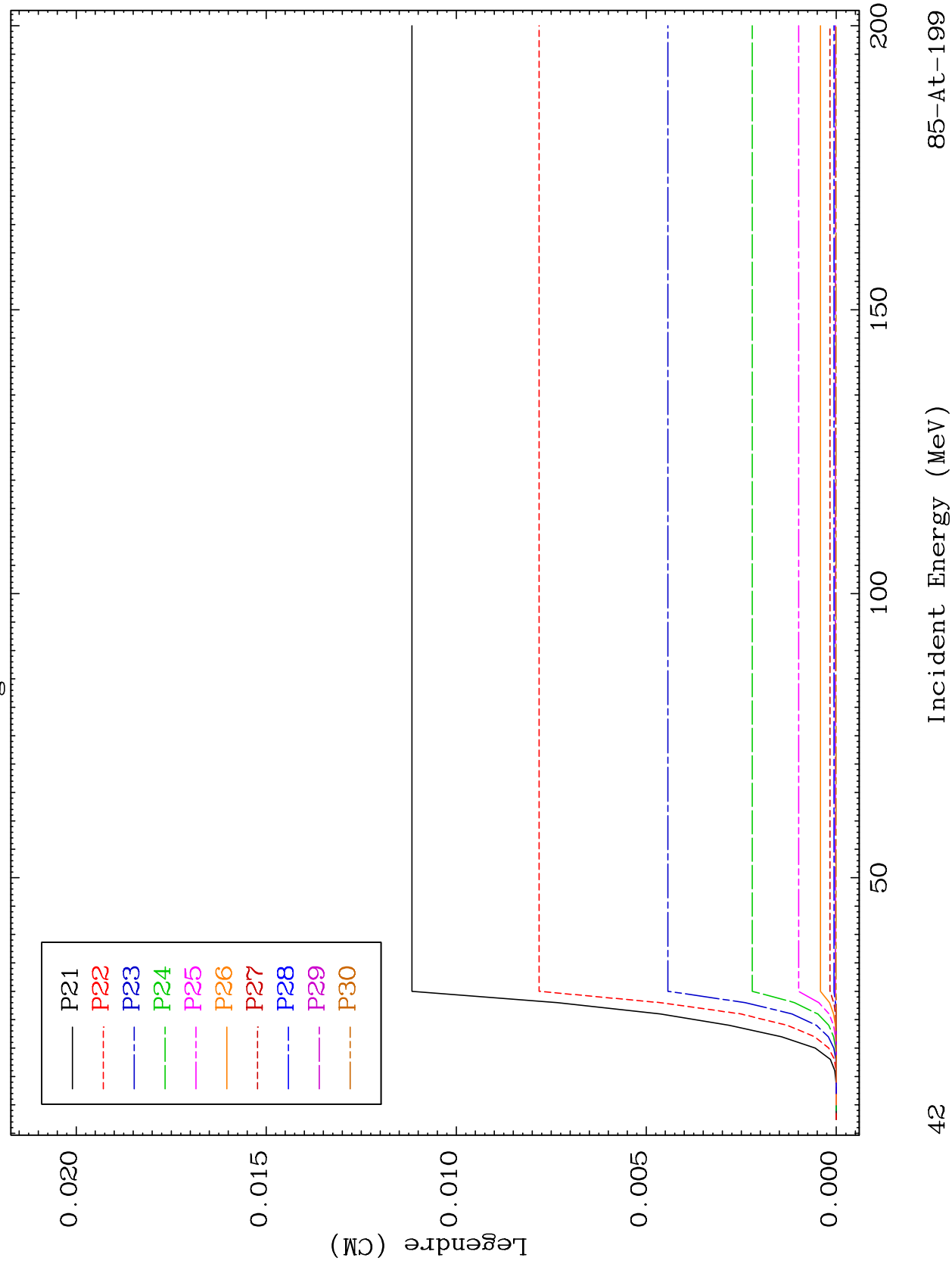


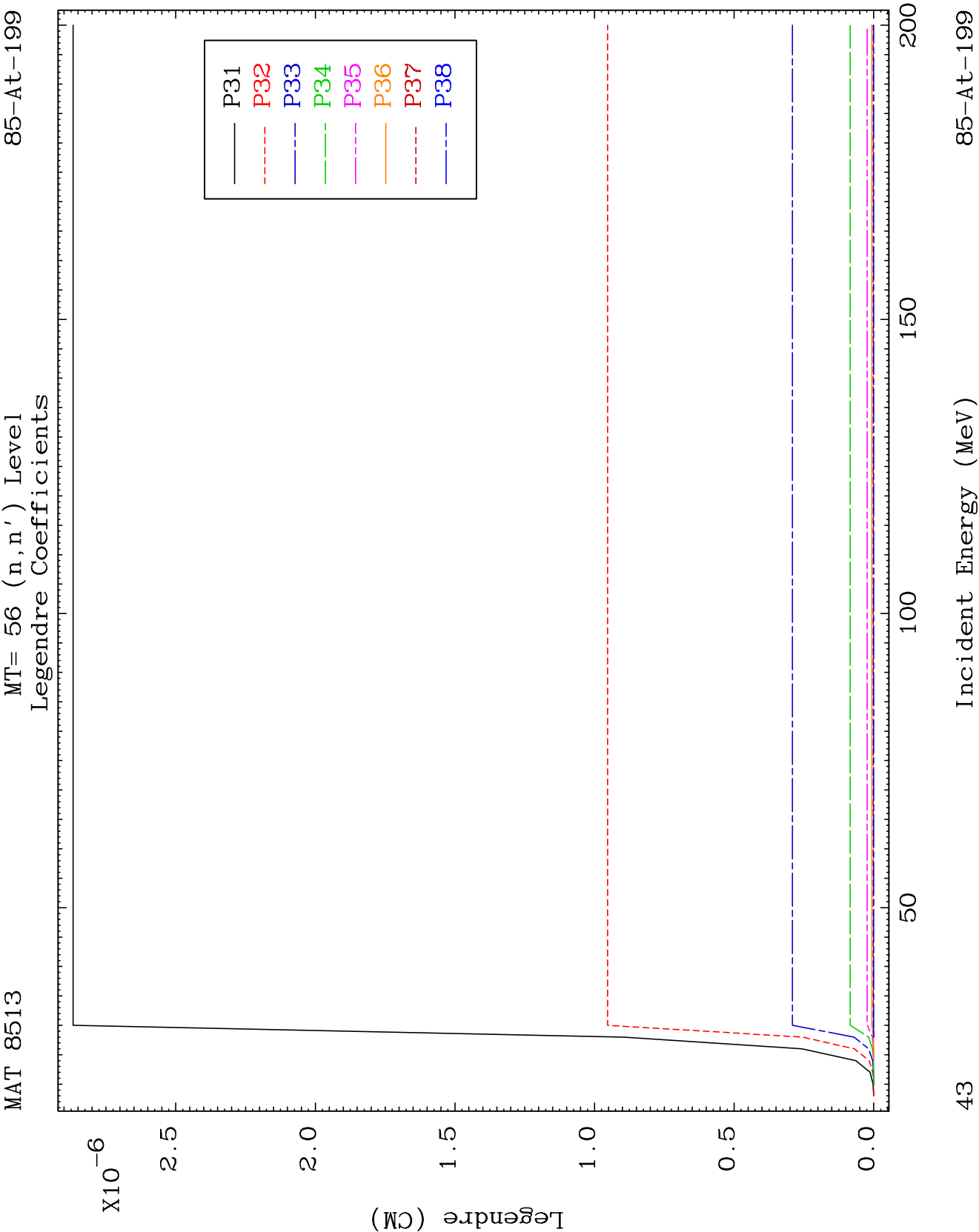


MAT 8513

MT= 56 (n, n') Level
Legendre Coefficients

85-At-199

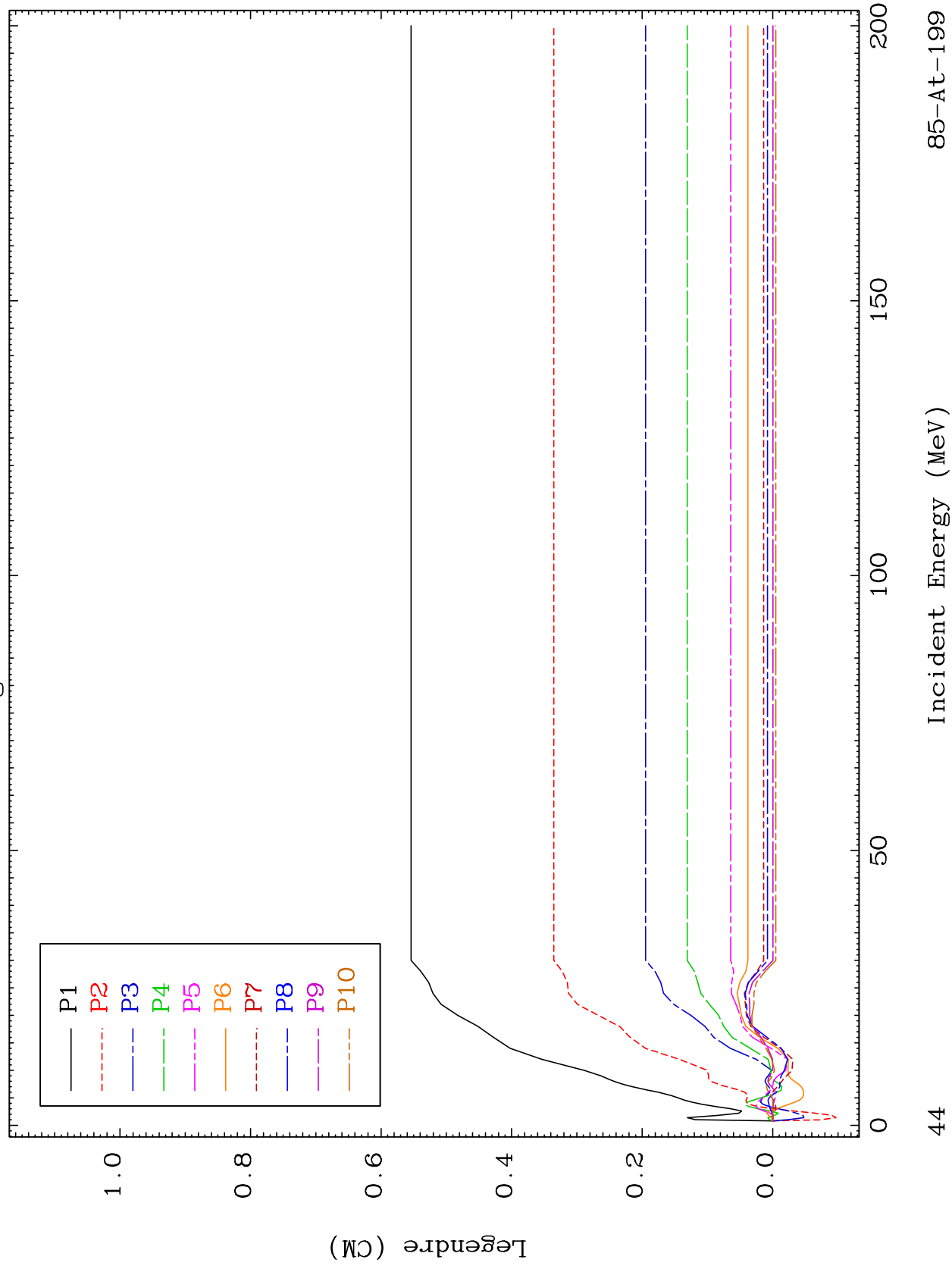


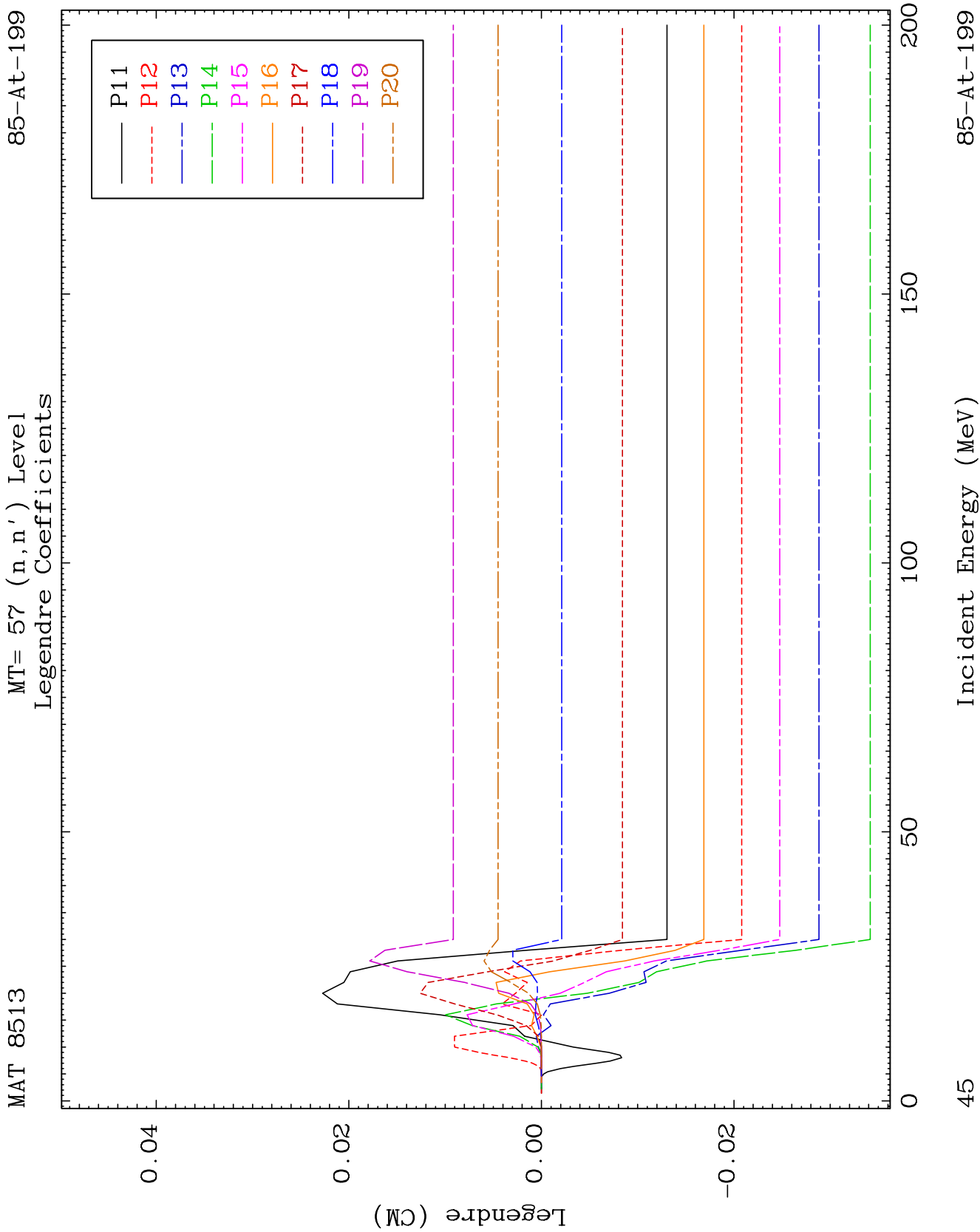


MAT 8513

MT= 57 (n, n') Level
Legendre Coefficients

85-At-199

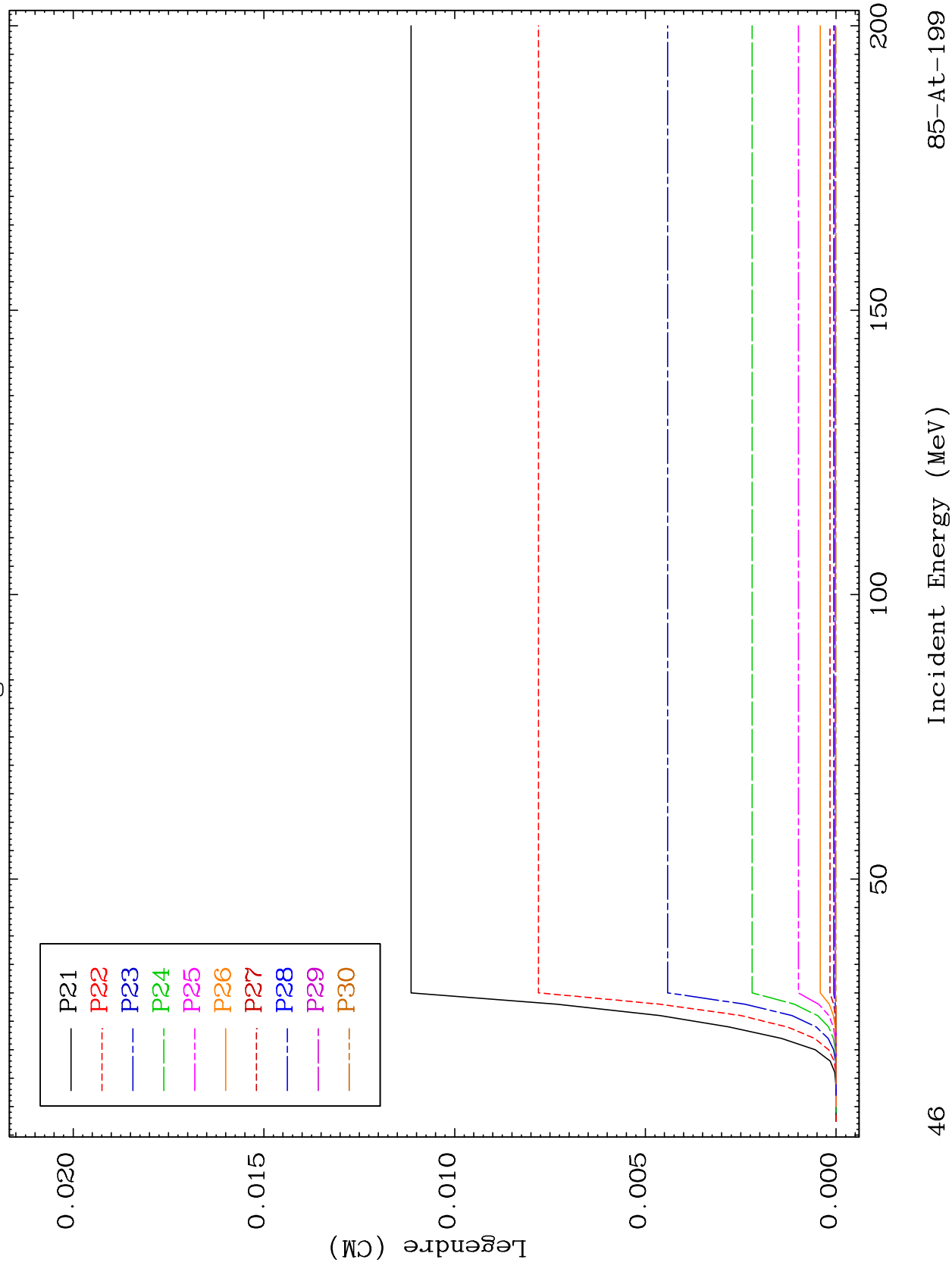


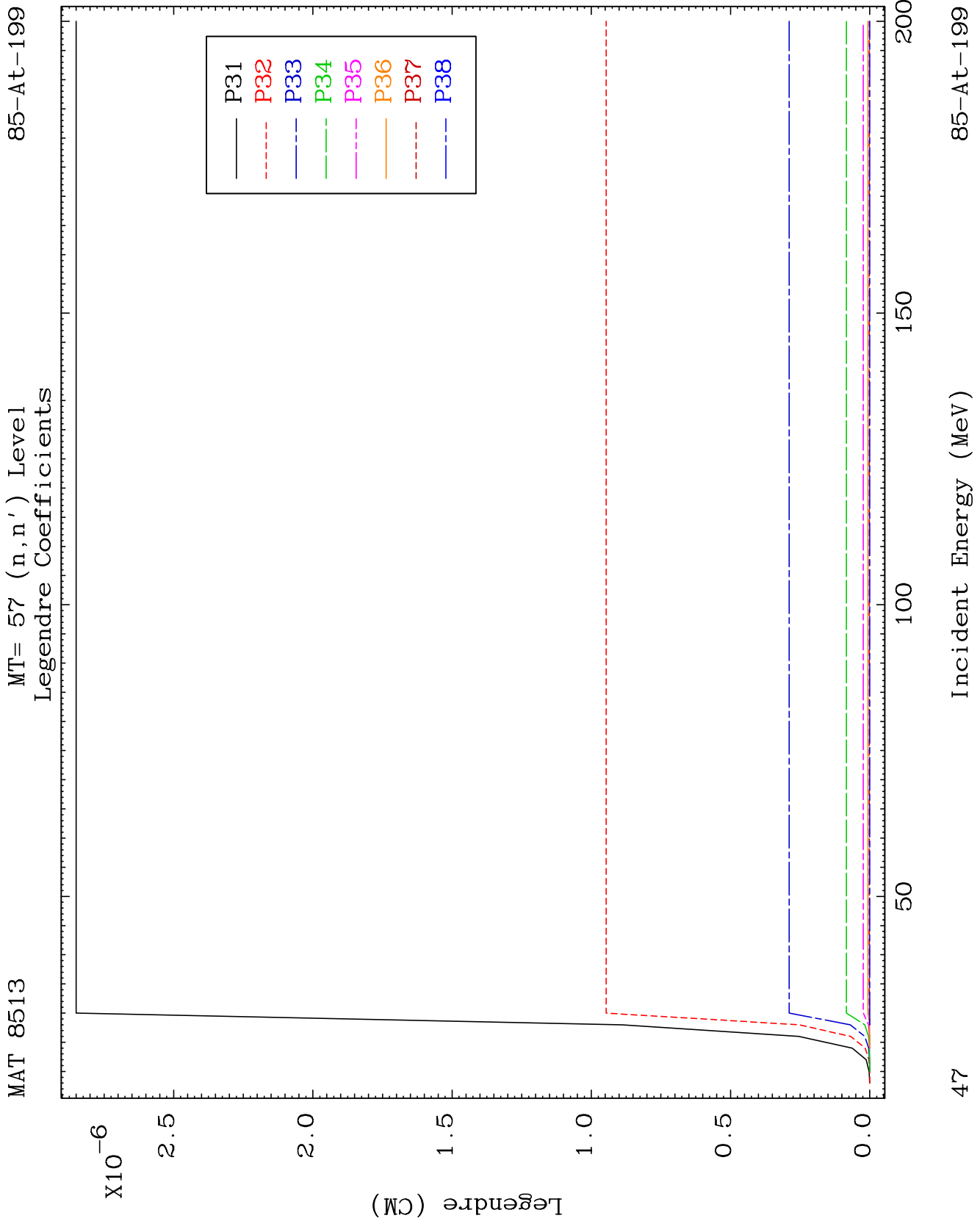


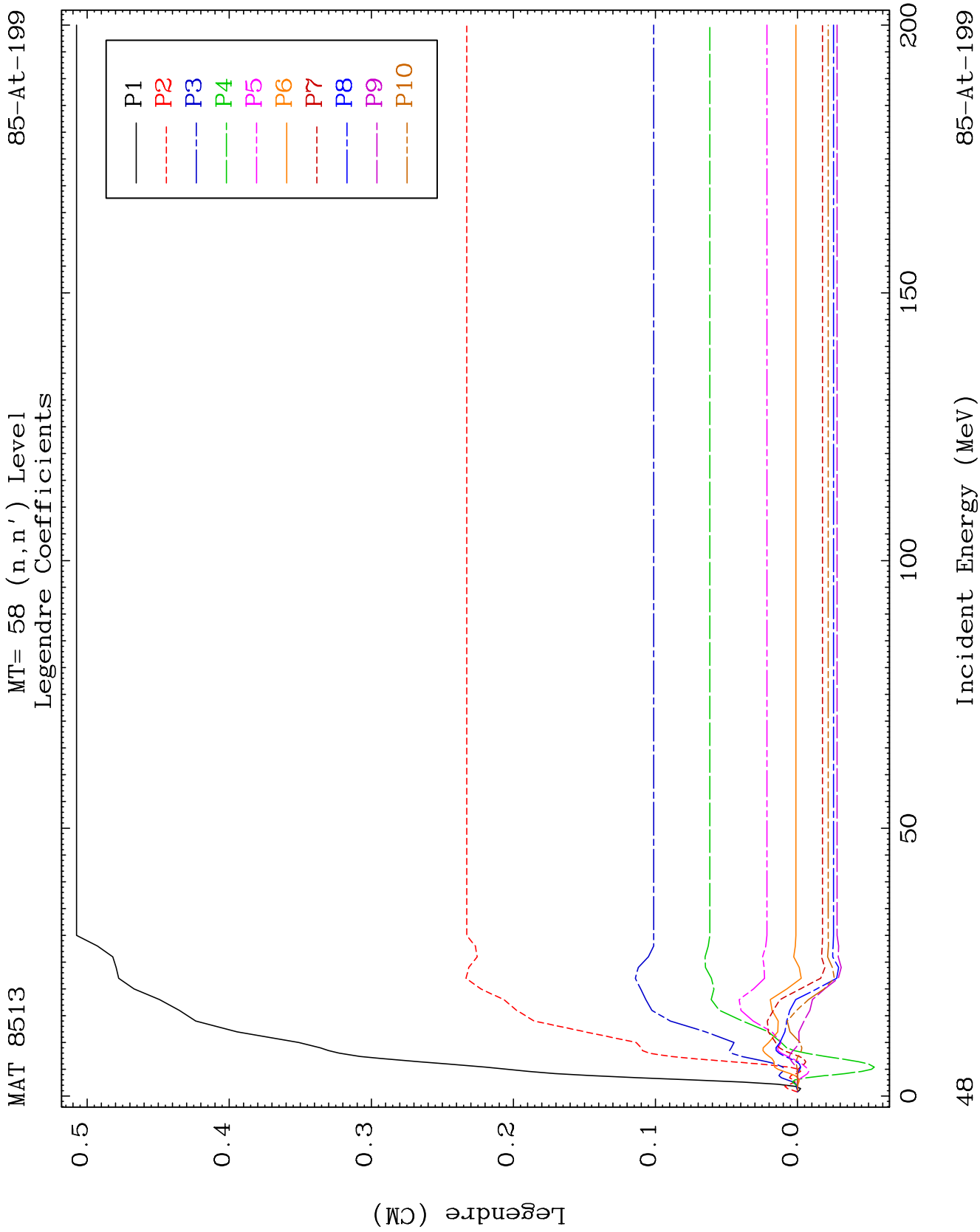
MAT 8513

MT= 57 (n, n') Level
Legendre Coefficients

85-At-199



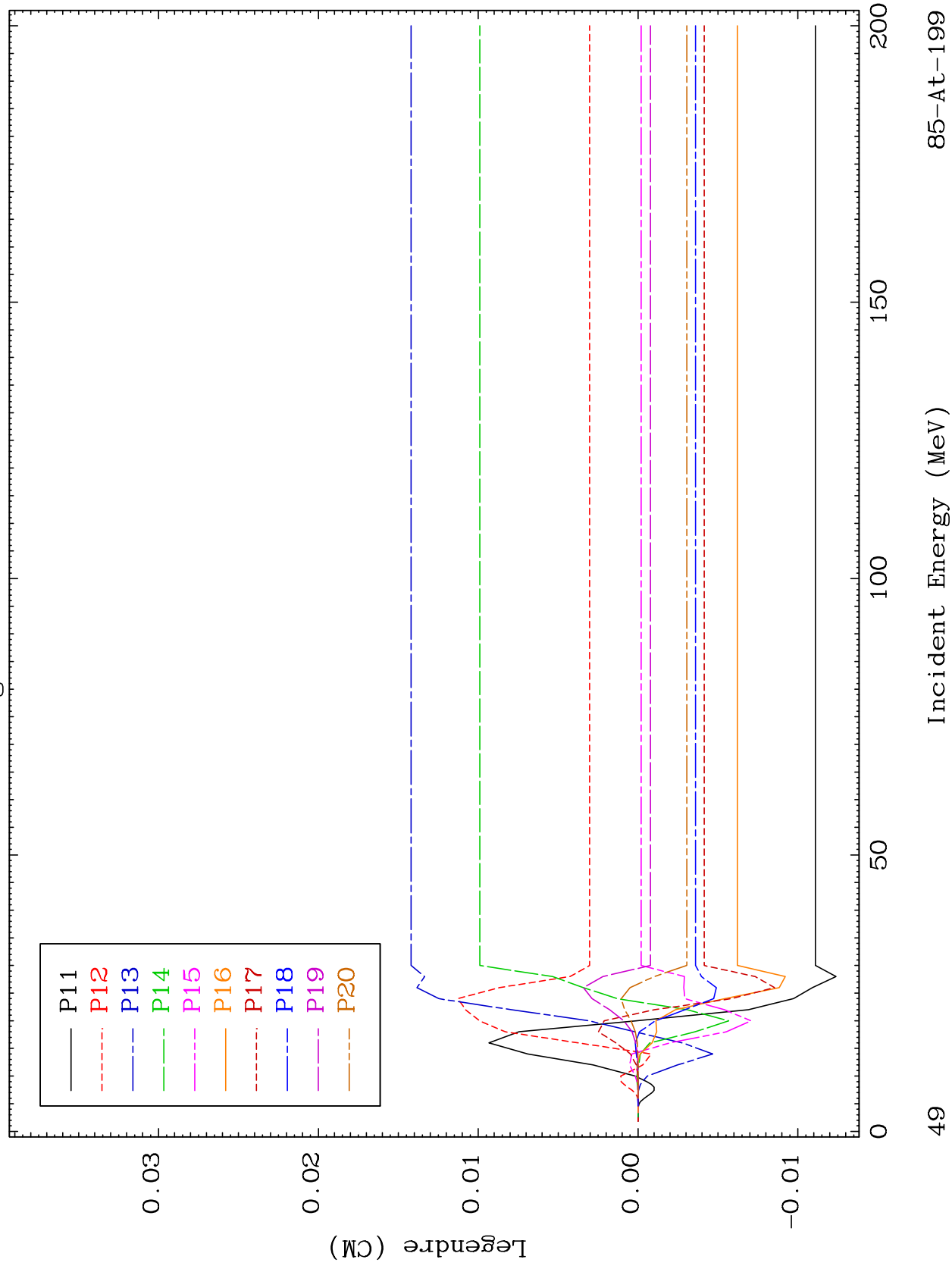




MAT 8513

MT= 58 (n, n') Level
Legendre Coefficients

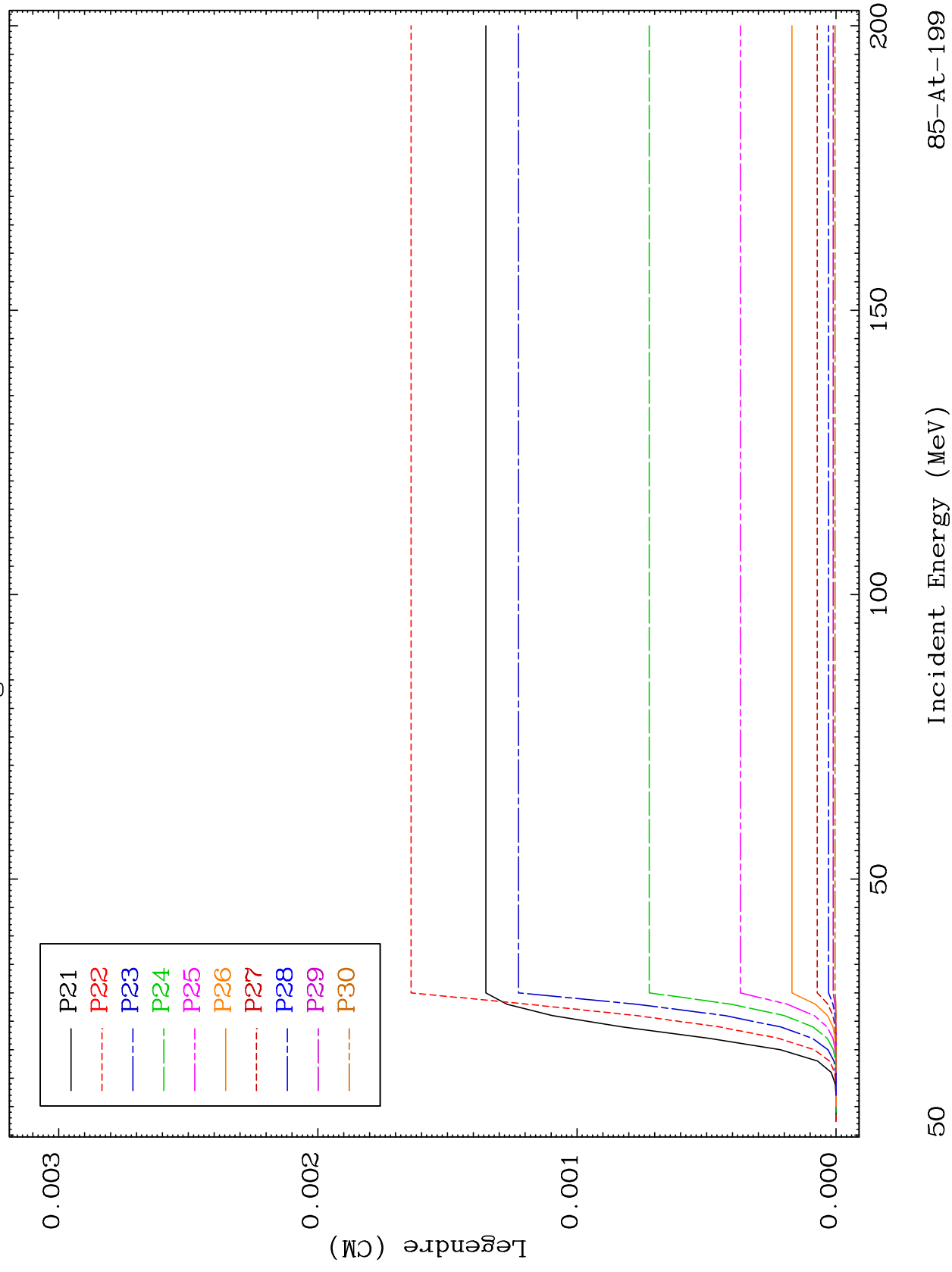
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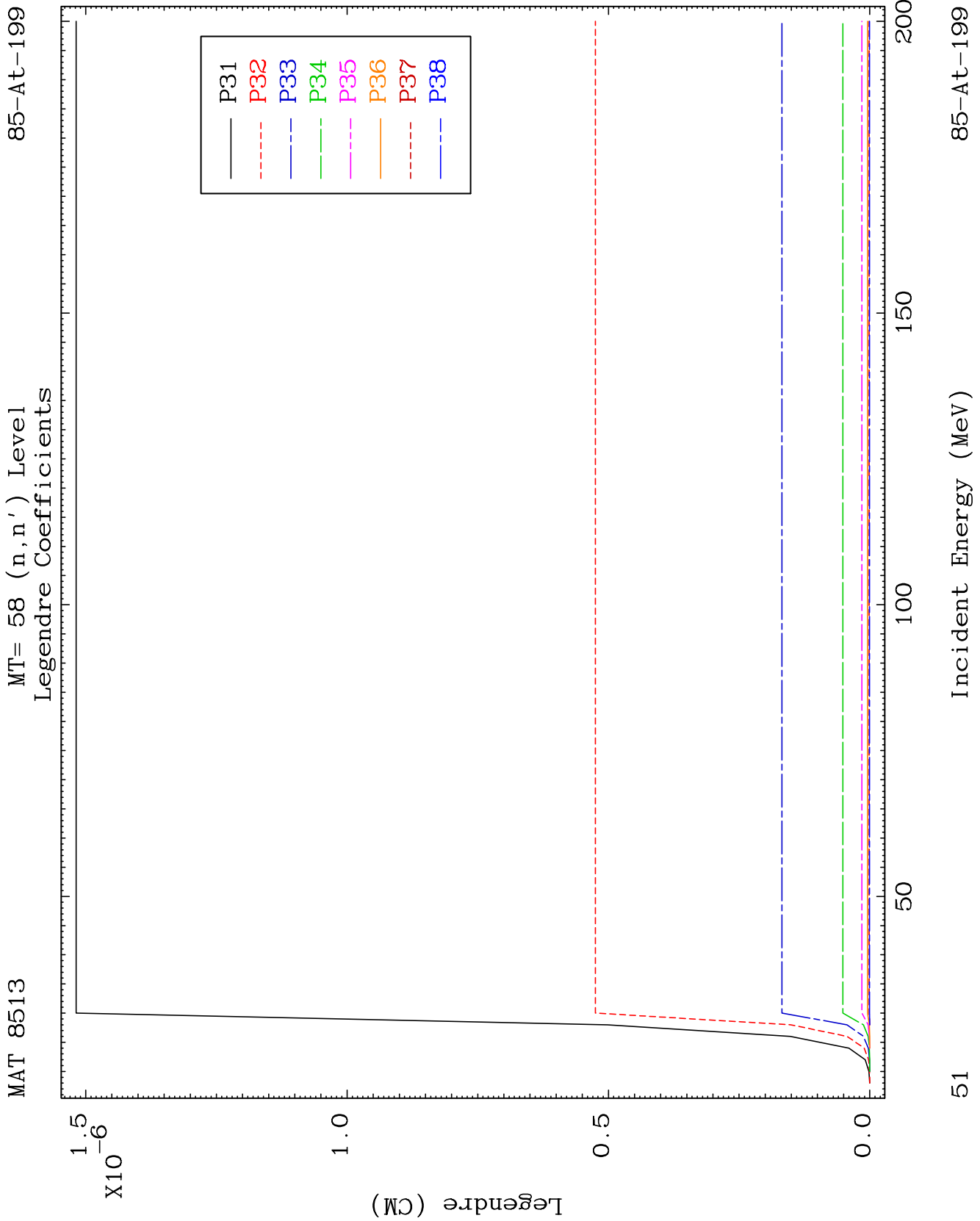


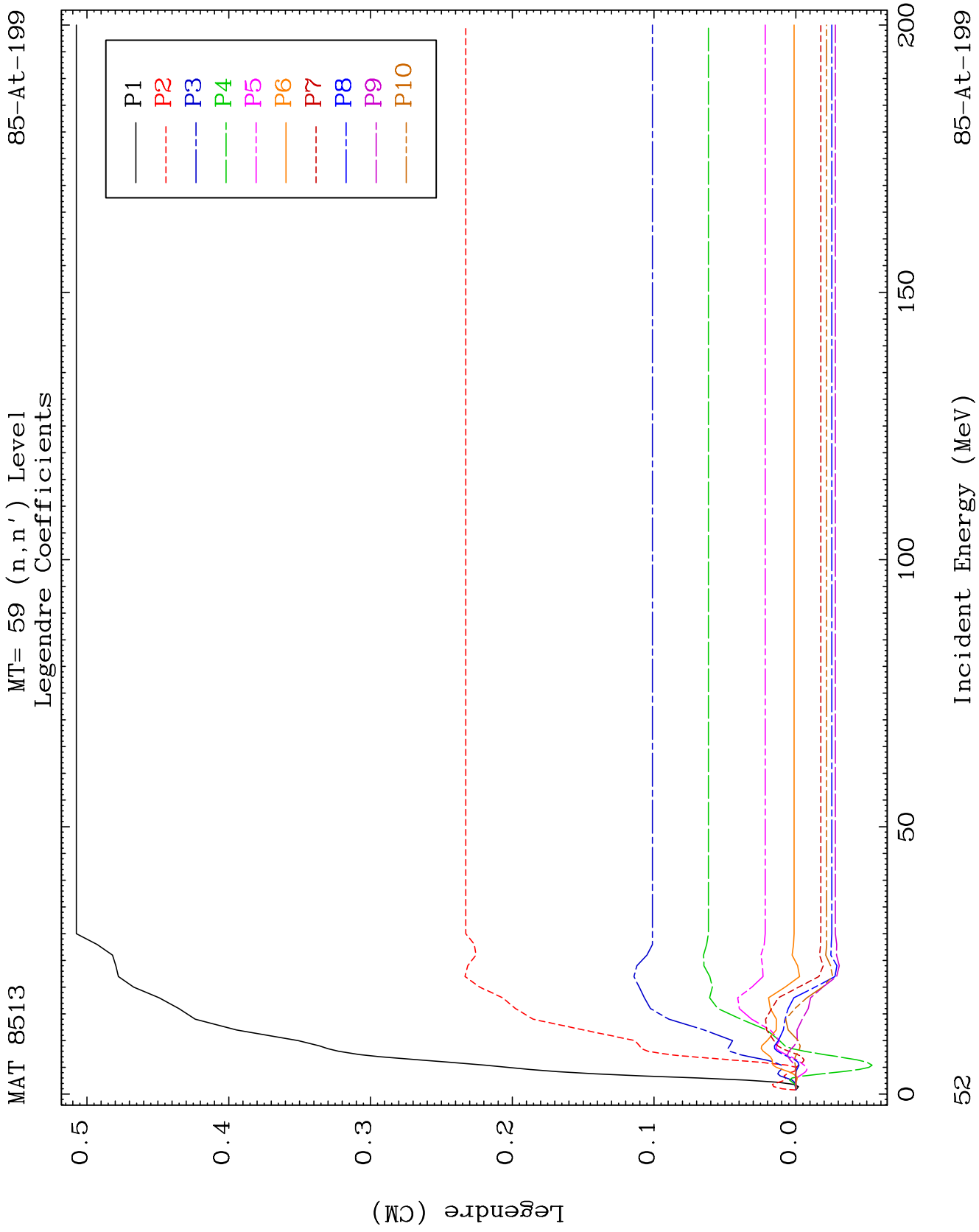
MAT 8513

MT= 58 (n, n') Level
Legendre Coefficients

85-At-199



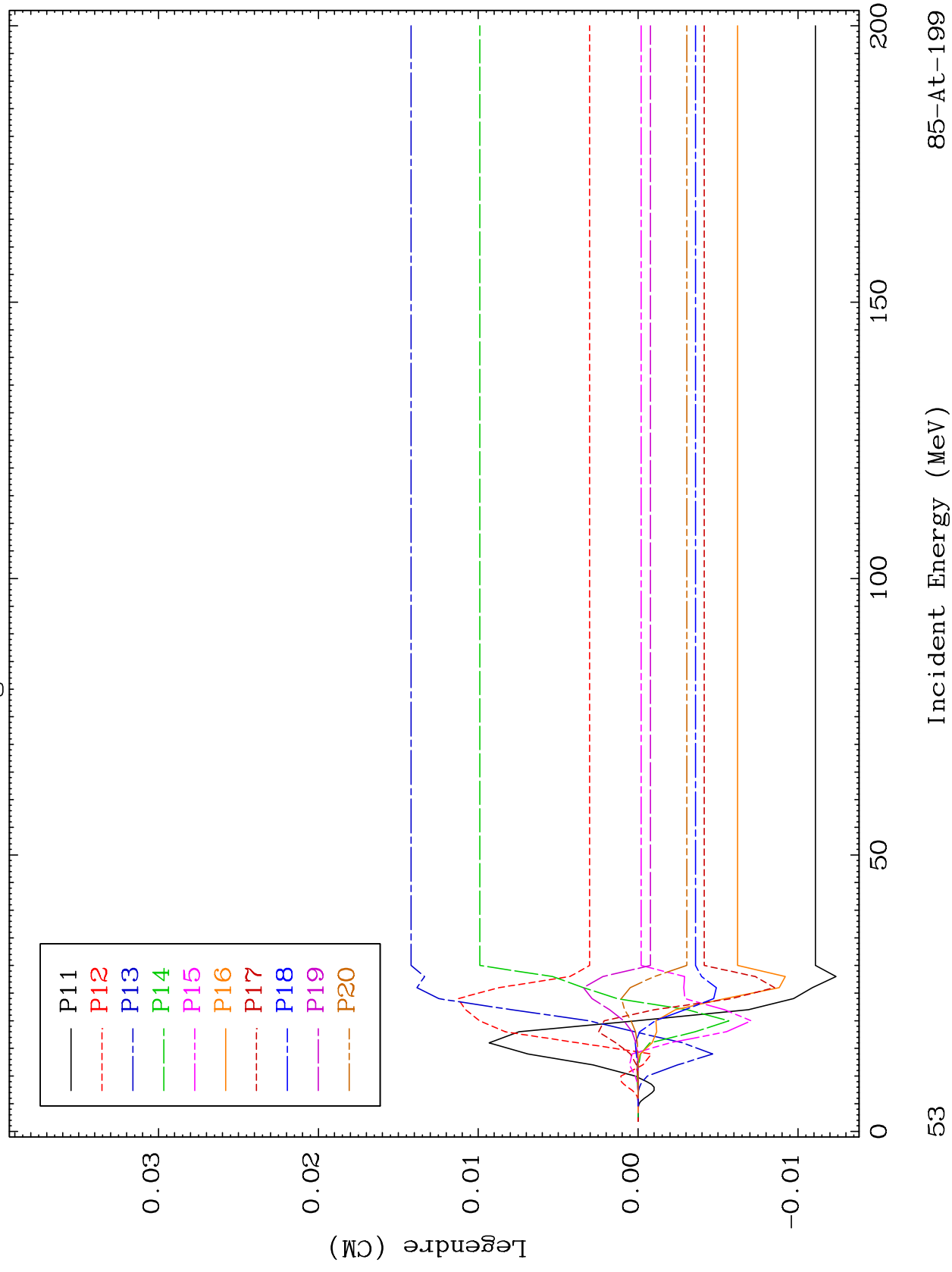




MAT 8513

MT= 59 (n, n') Level
Legendre Coefficients

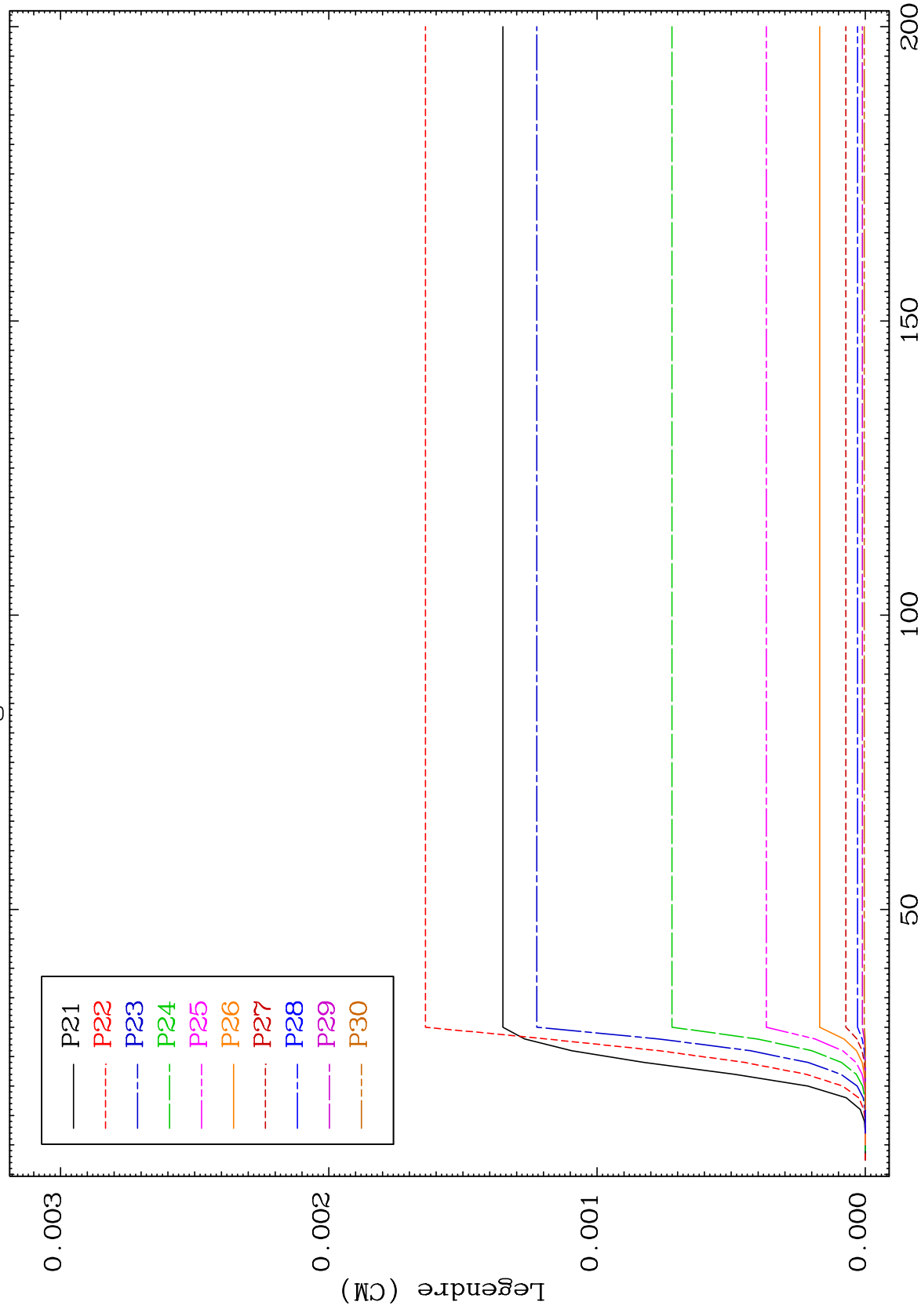
85-At-199



MAT 8513

MT= 59 (n, n') Level
Legendre Coefficients

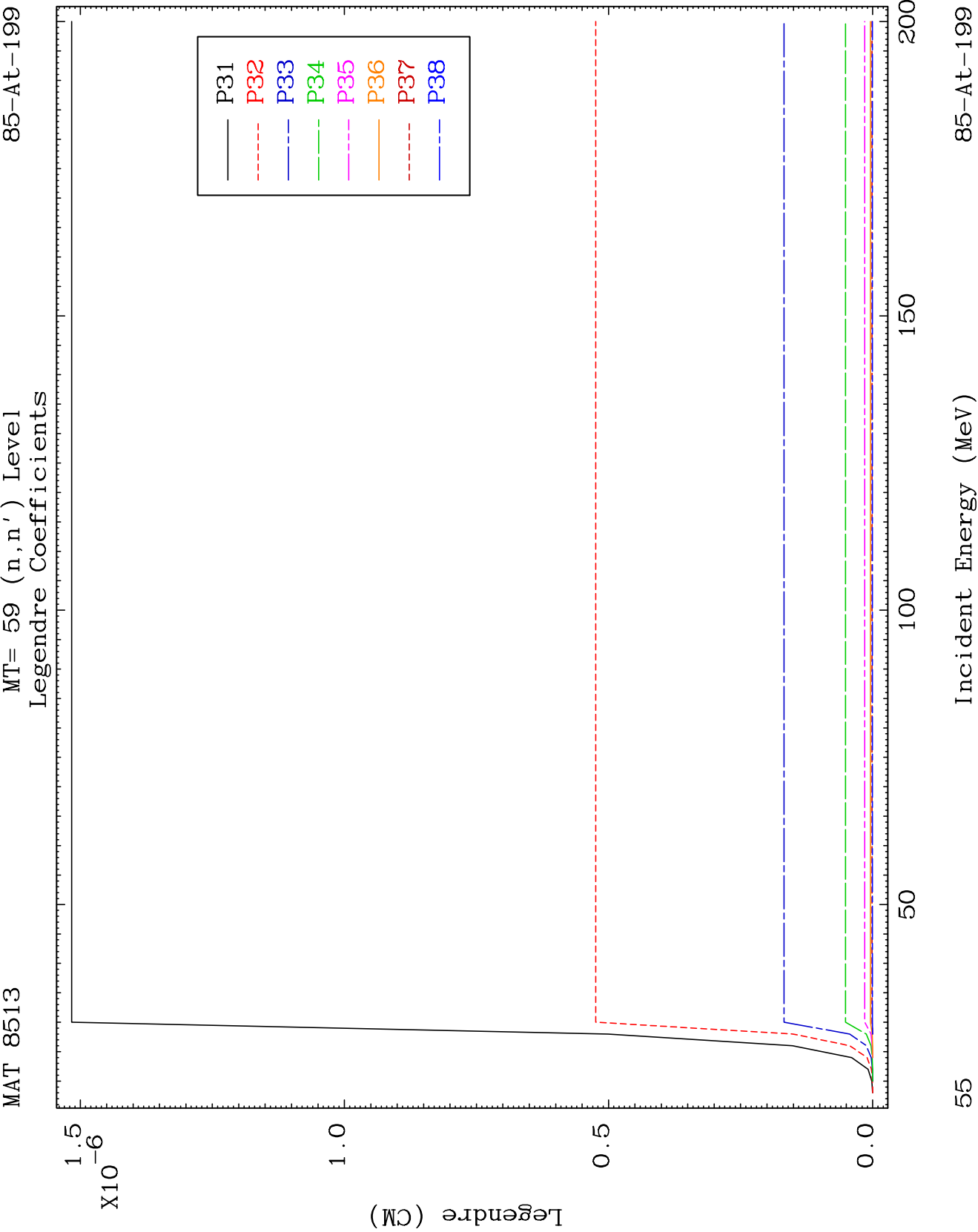
85-At-199

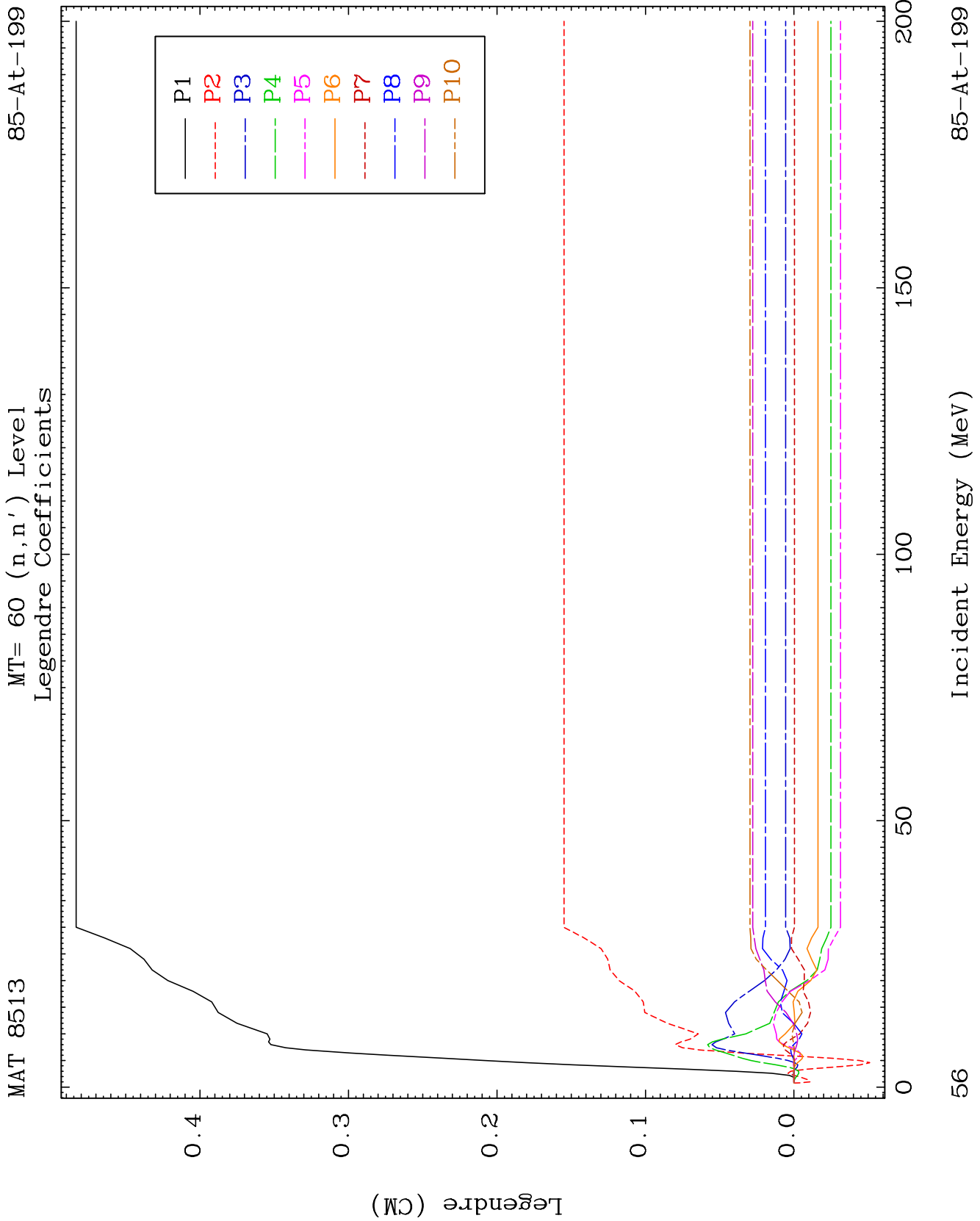


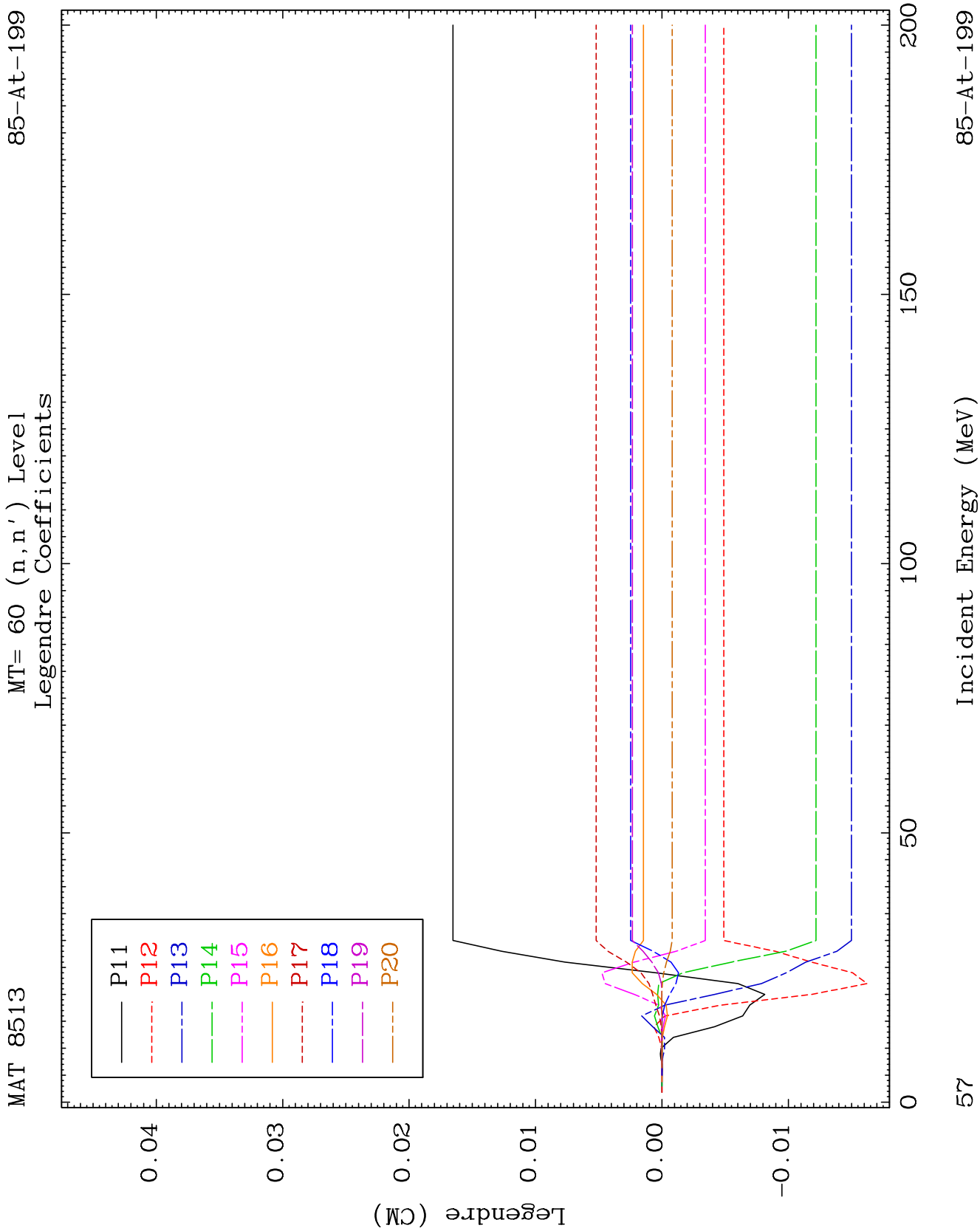
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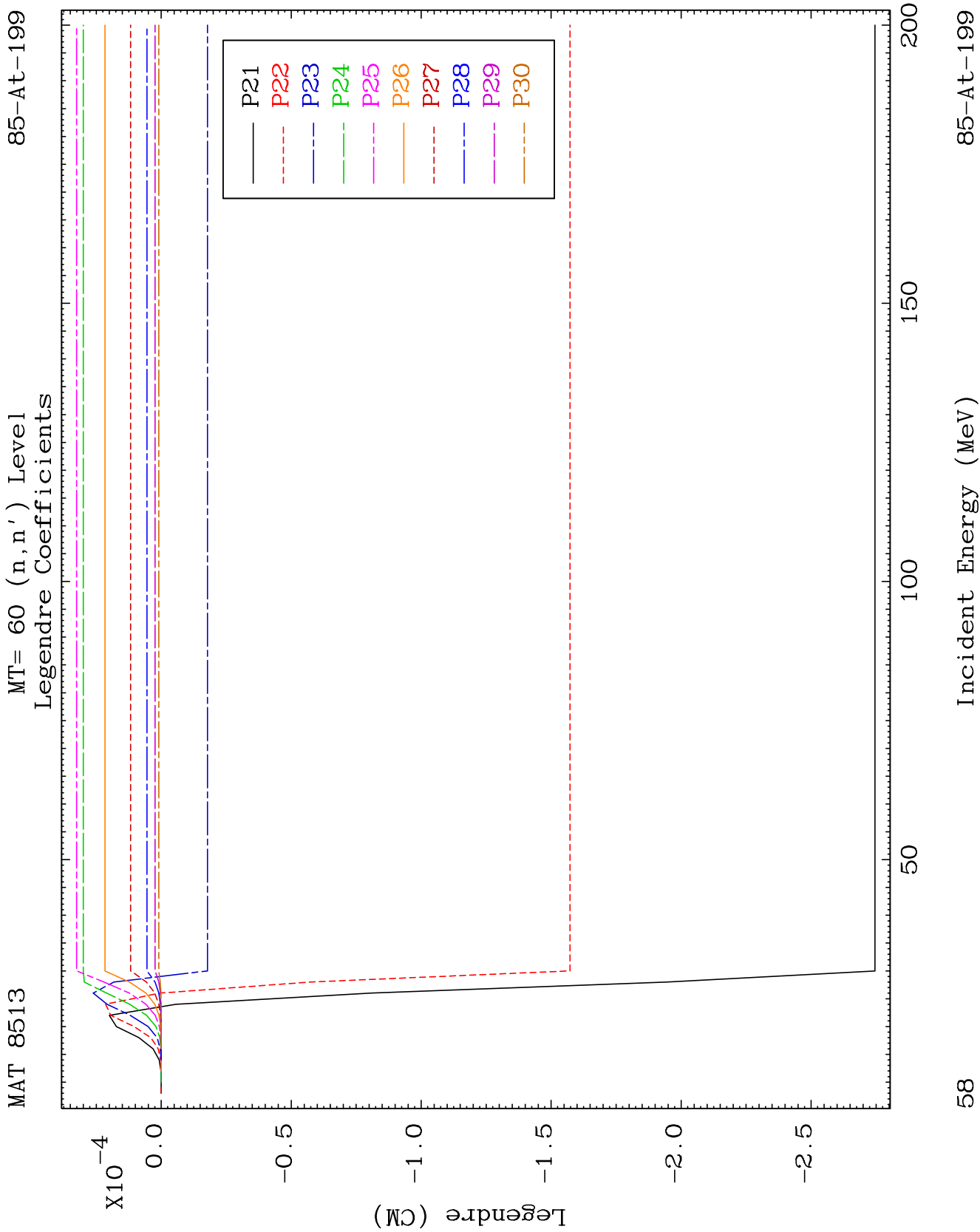
Incident Energy (MeV)

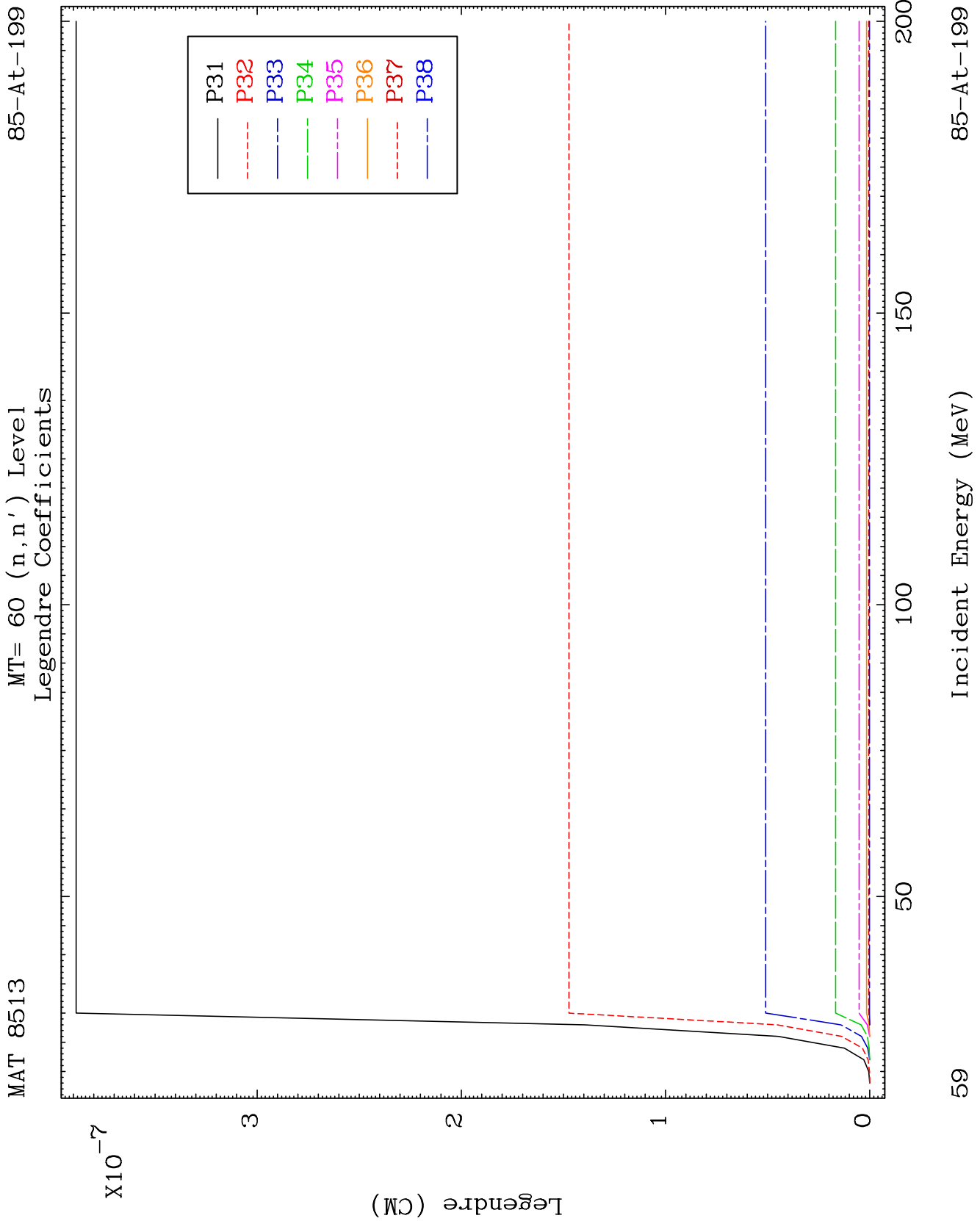
85-At-199







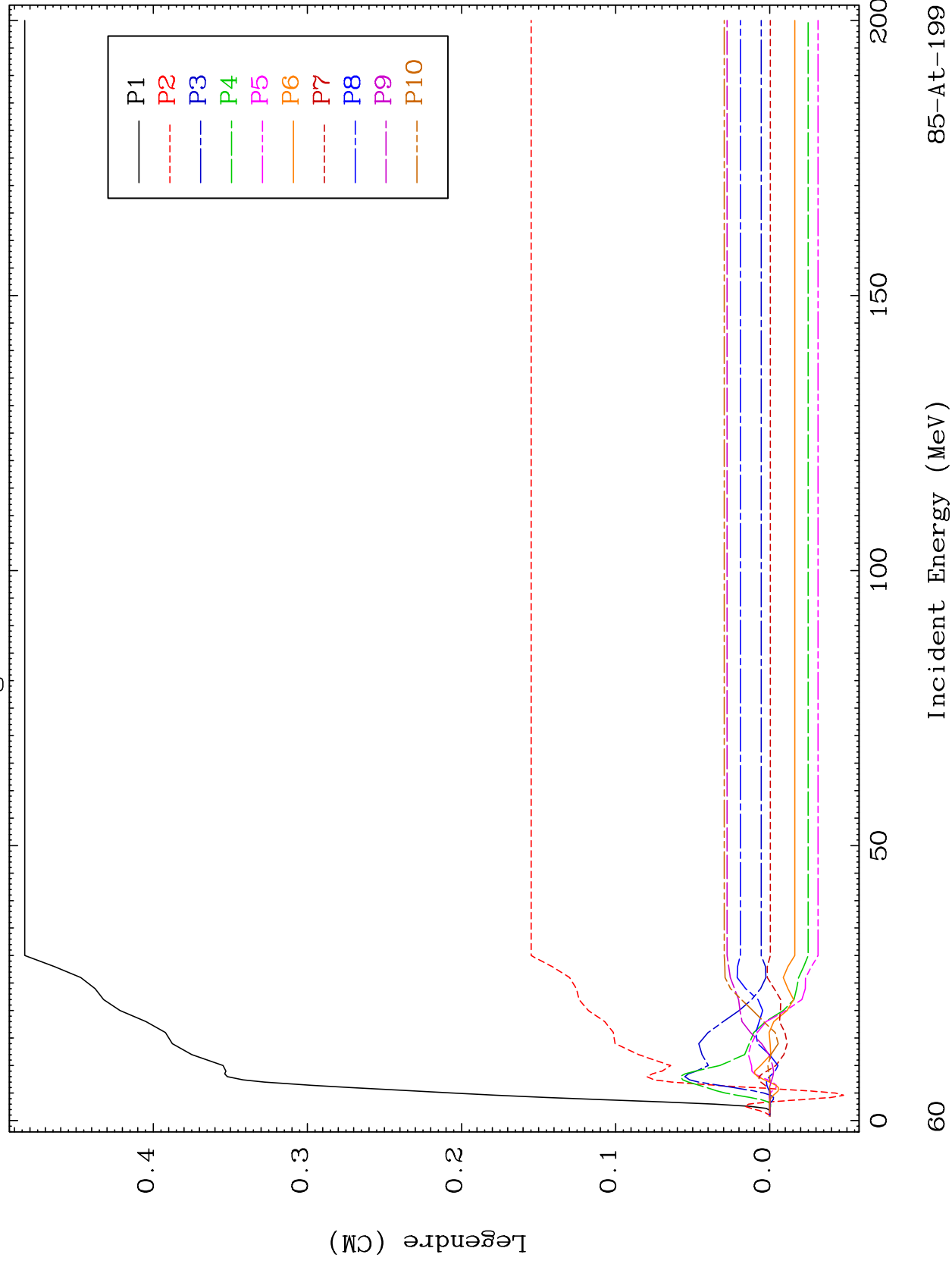




MAT 8513

MT= 61 (n, n') Level
Legendre Coefficients

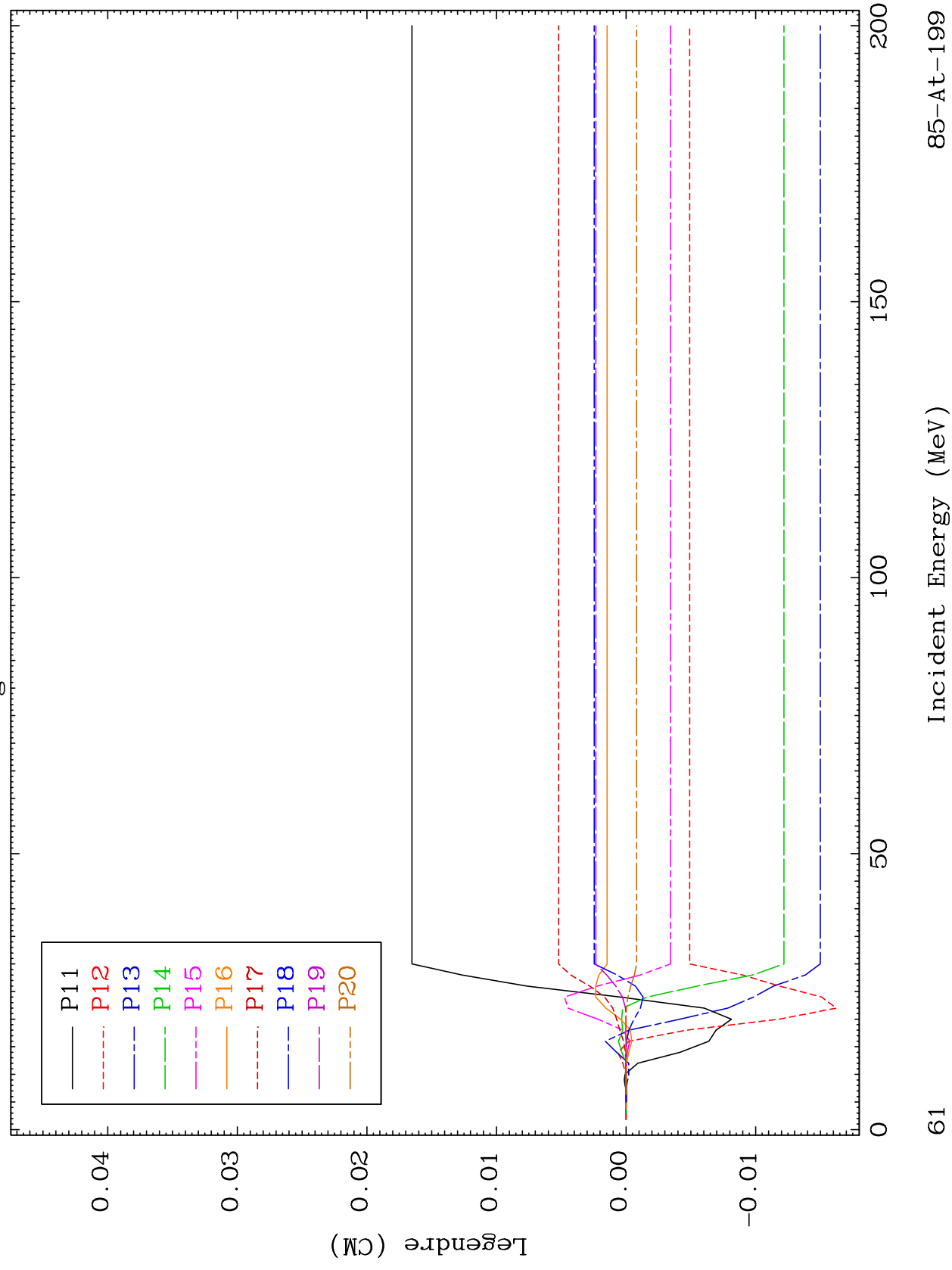
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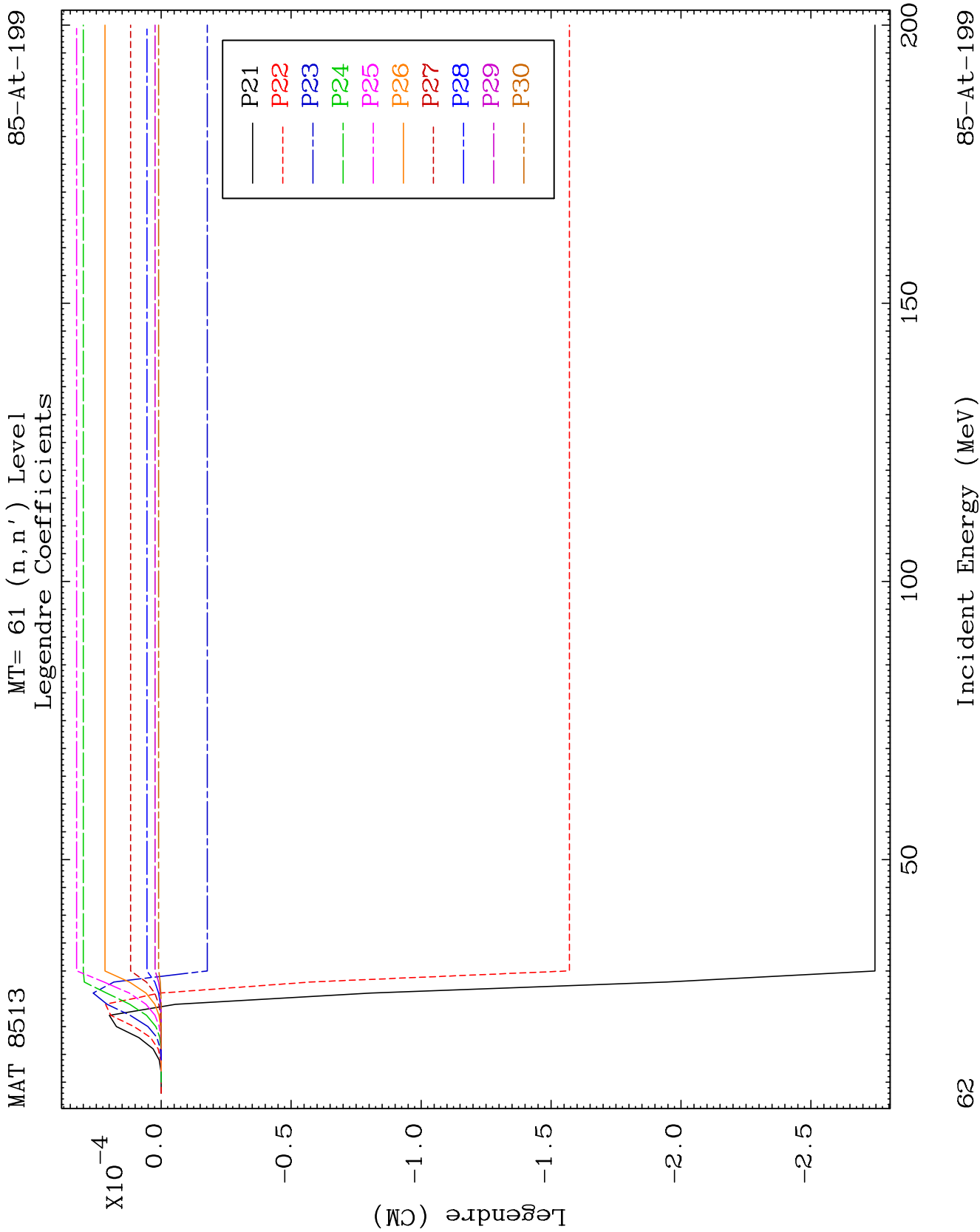


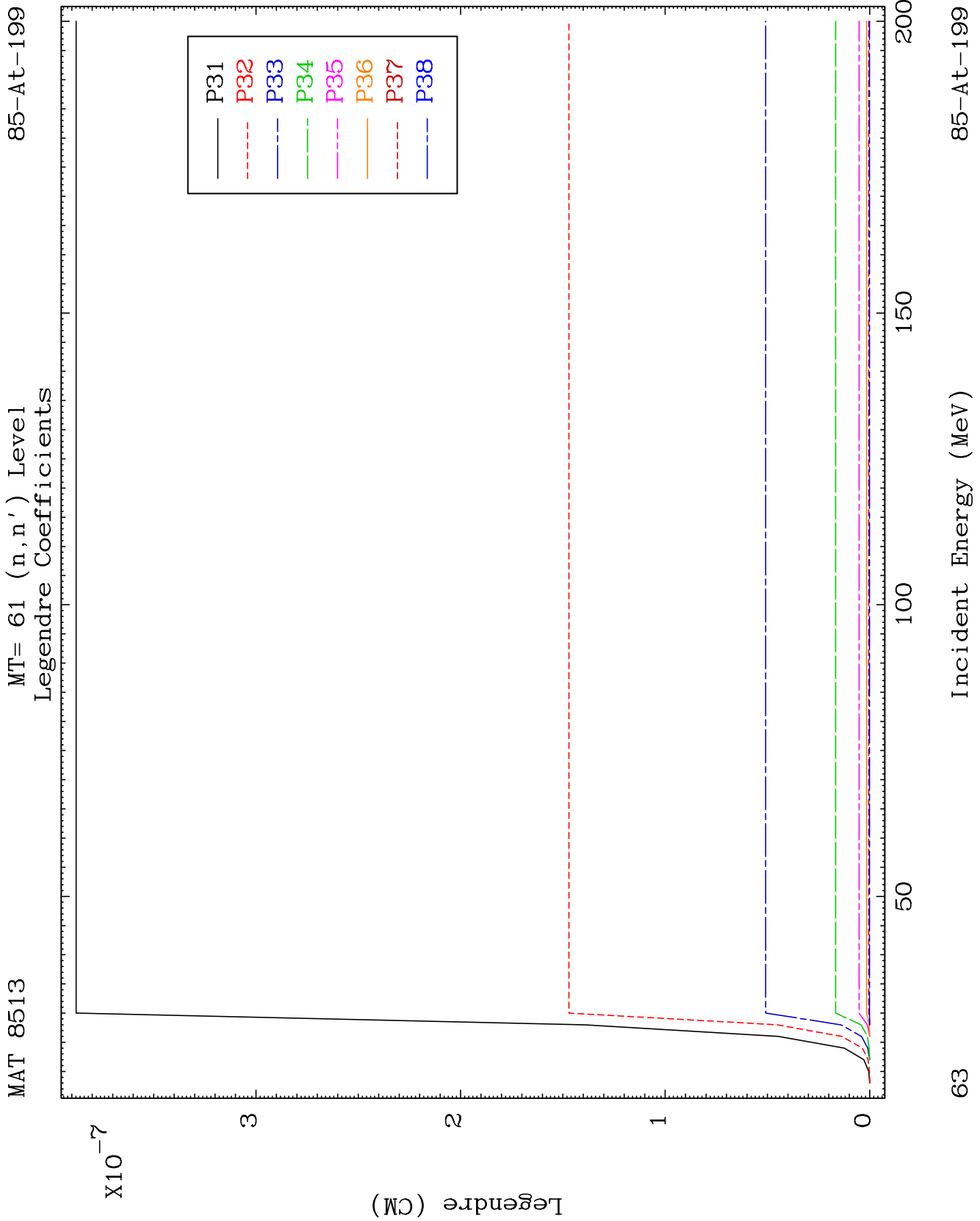
MAT 8513

MT= 61 (n, n') Level
Legendre Coefficients

85-At-199



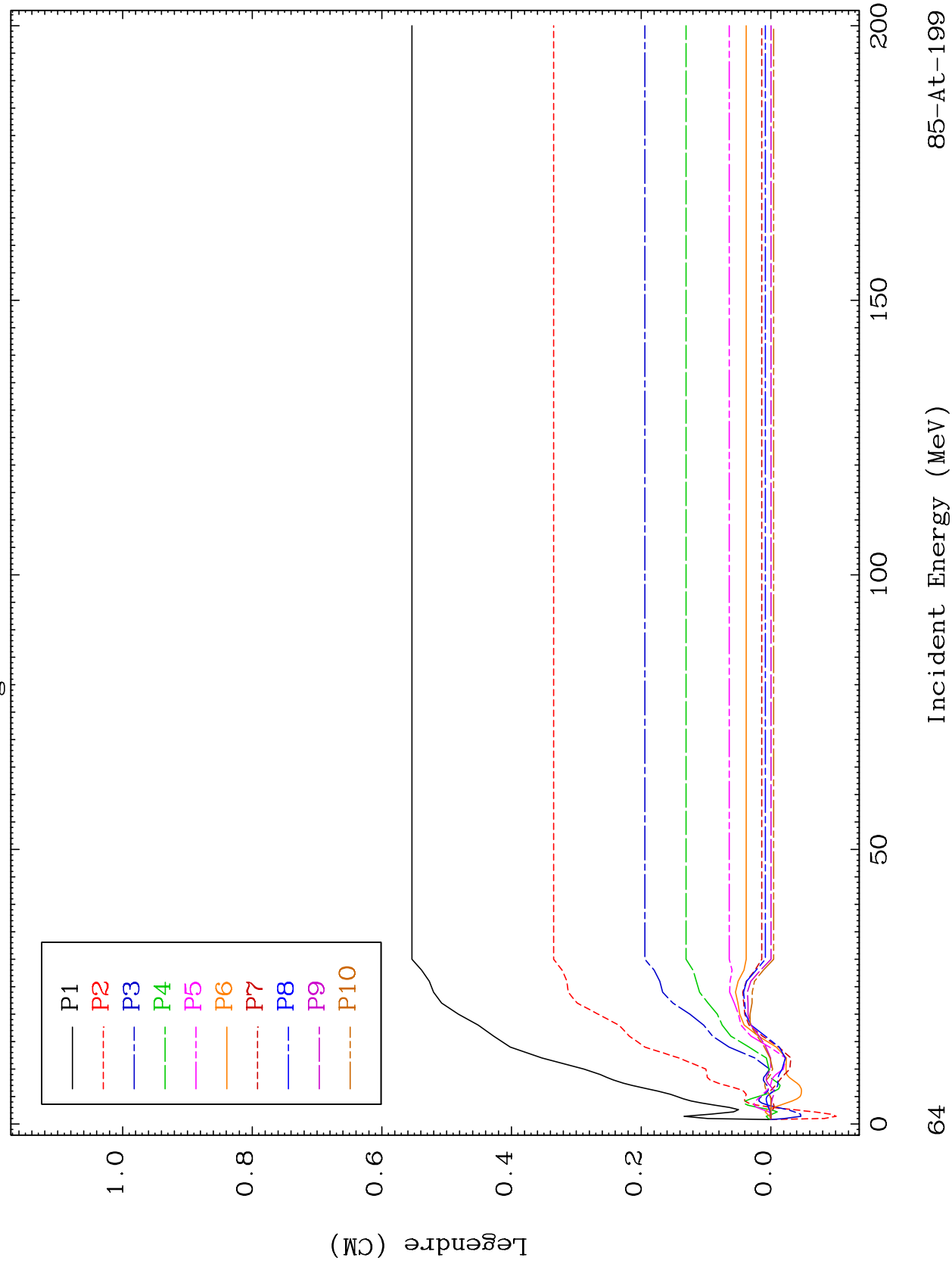


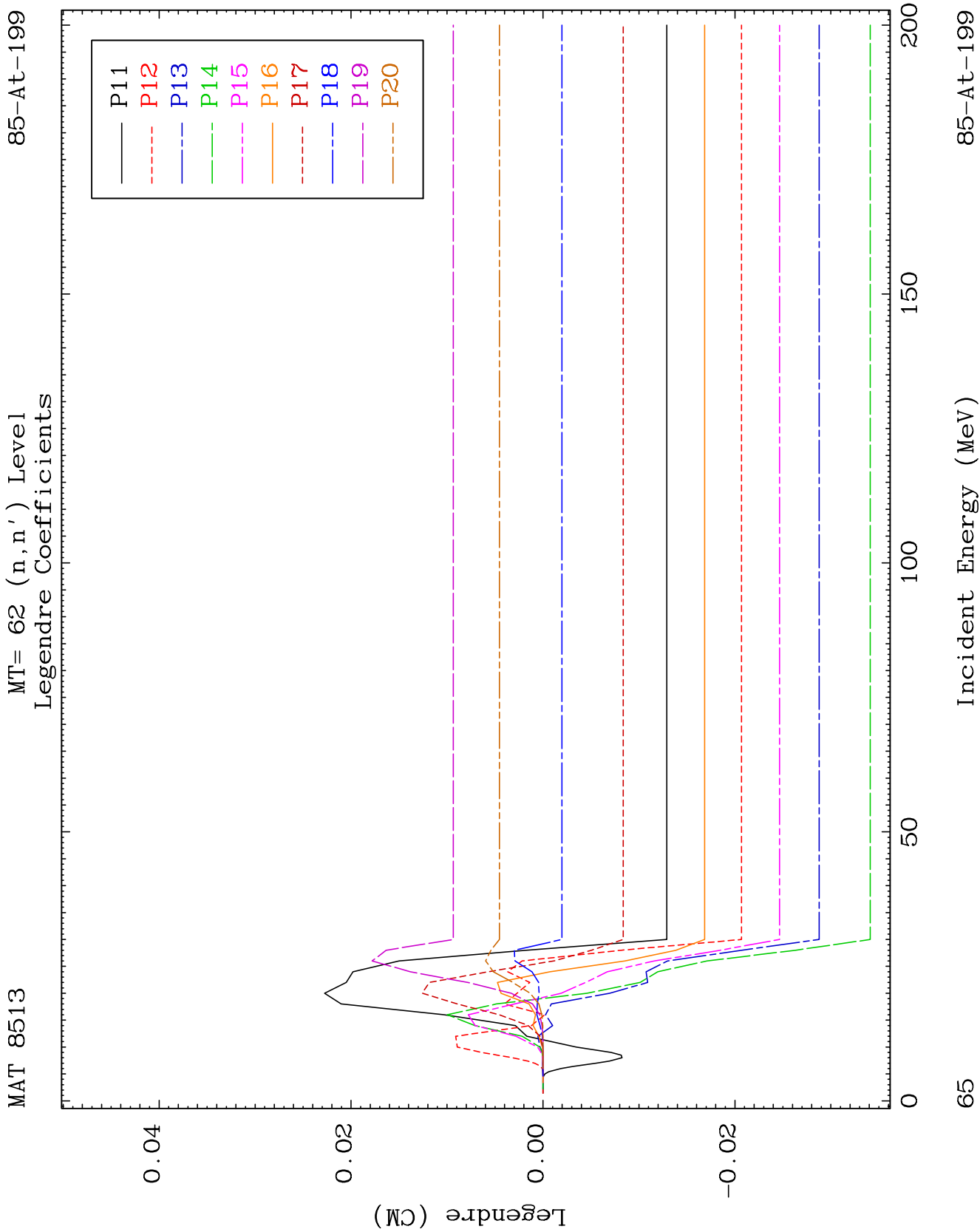


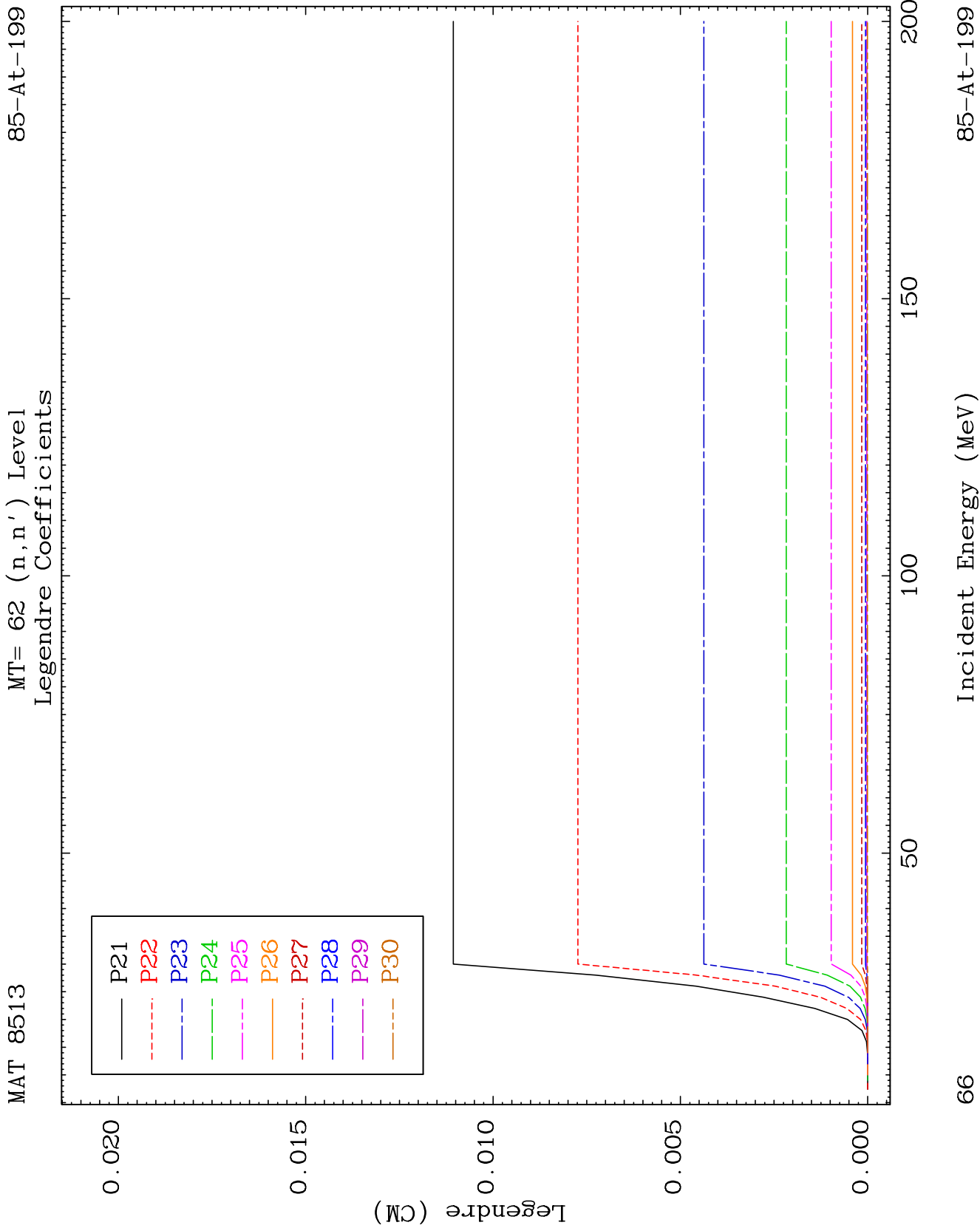
MAT 8513

MT= 62 (n, n') Level
Legendre Coefficients

85-At-199



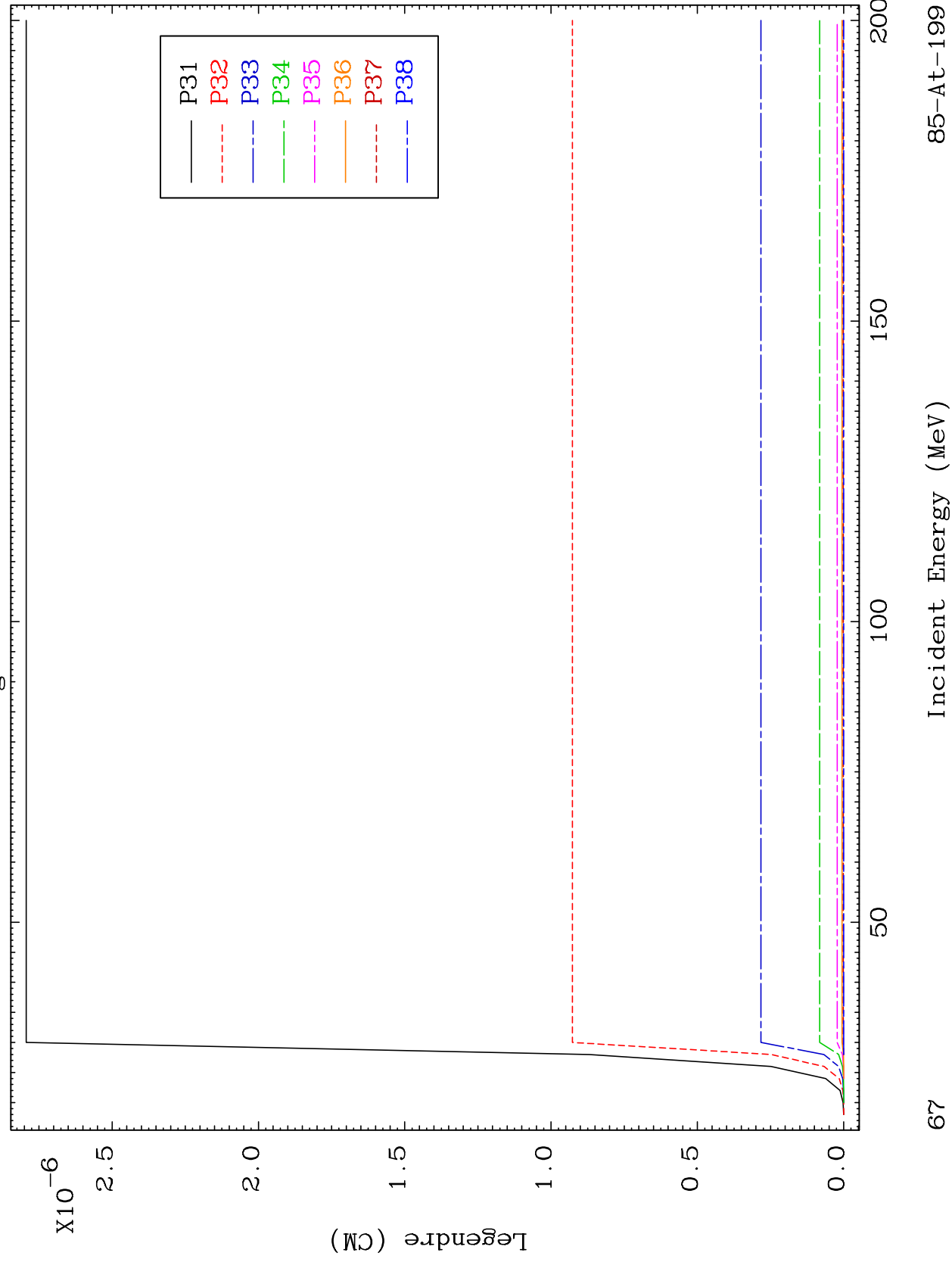


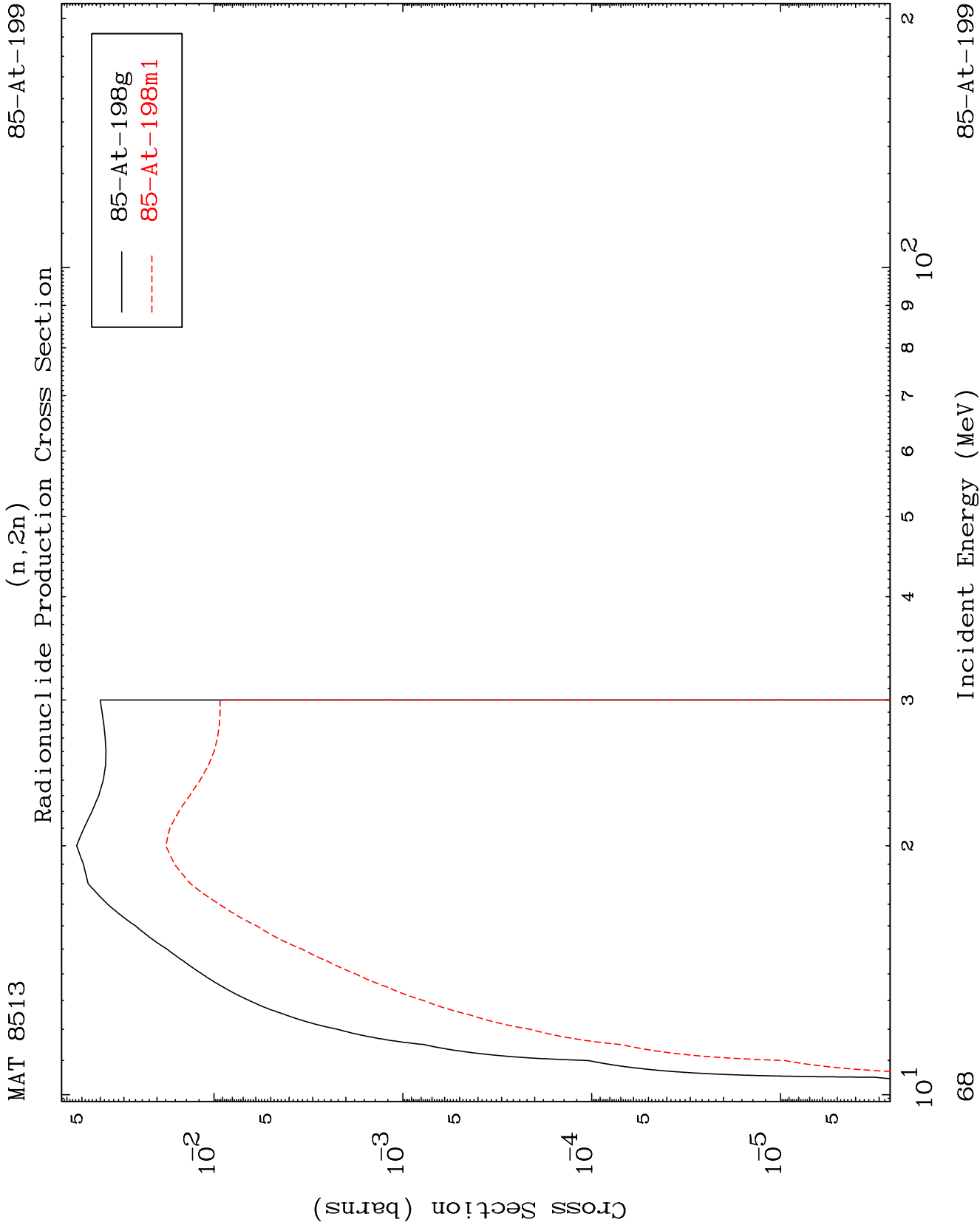


MAT 8513

MT= 62 (n, n') Level
Legendre Coefficients

85-At-199



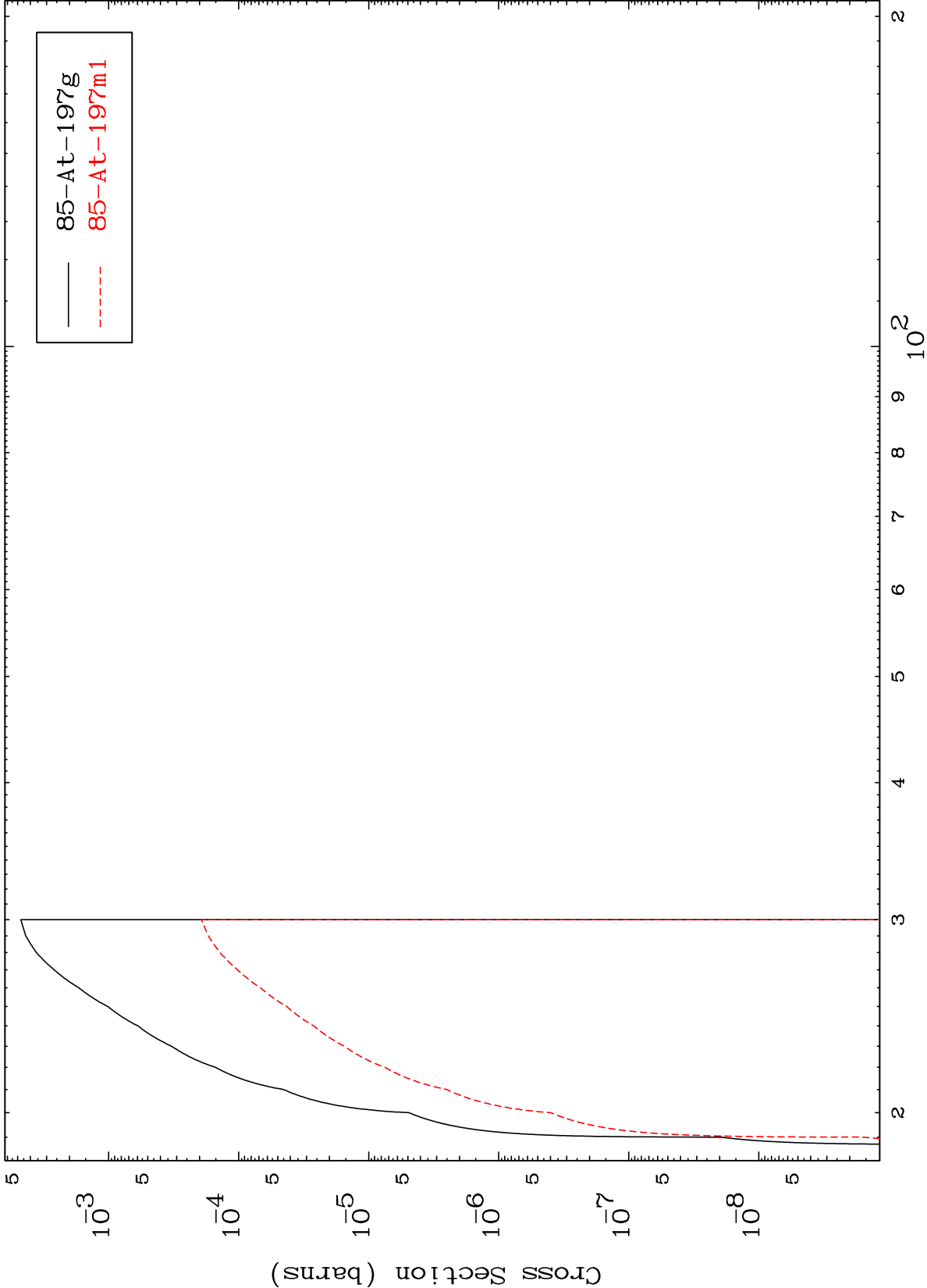


MAT 8513

(n,3n)

85-At-199

Radionuclide Production Cross Section



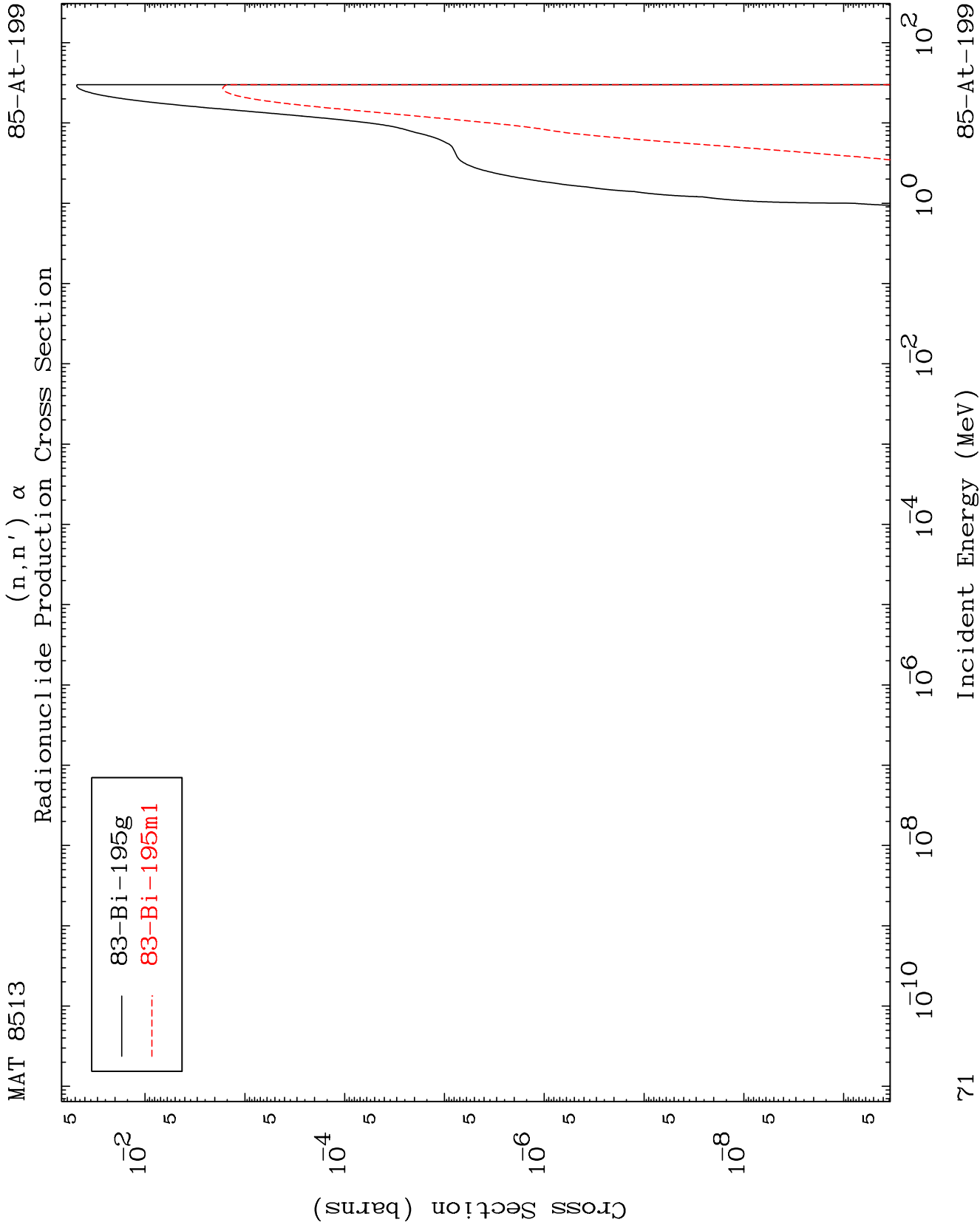
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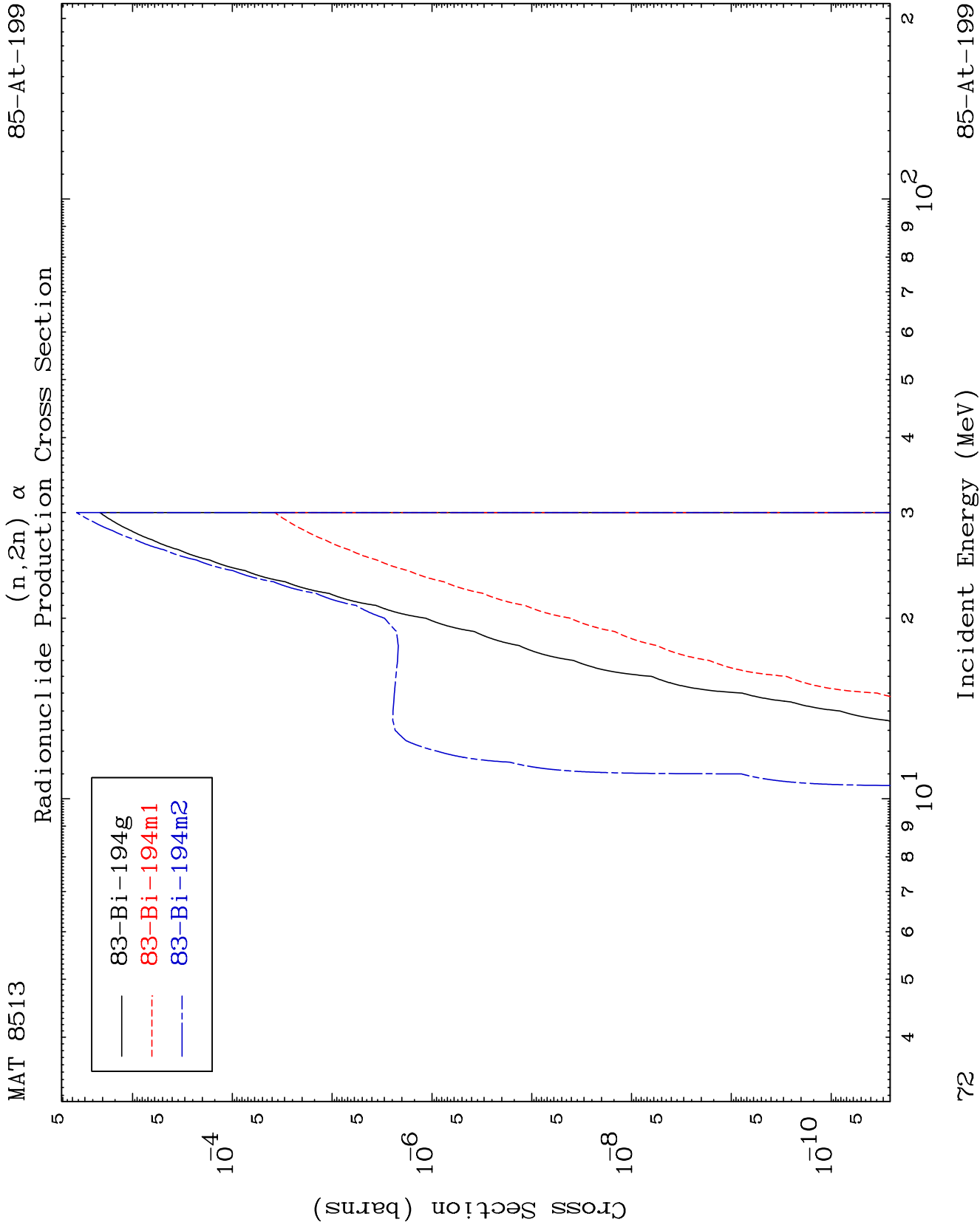
Incident Energy (MeV)

85-At-199

Fission
Radionuclide Production Cross Section





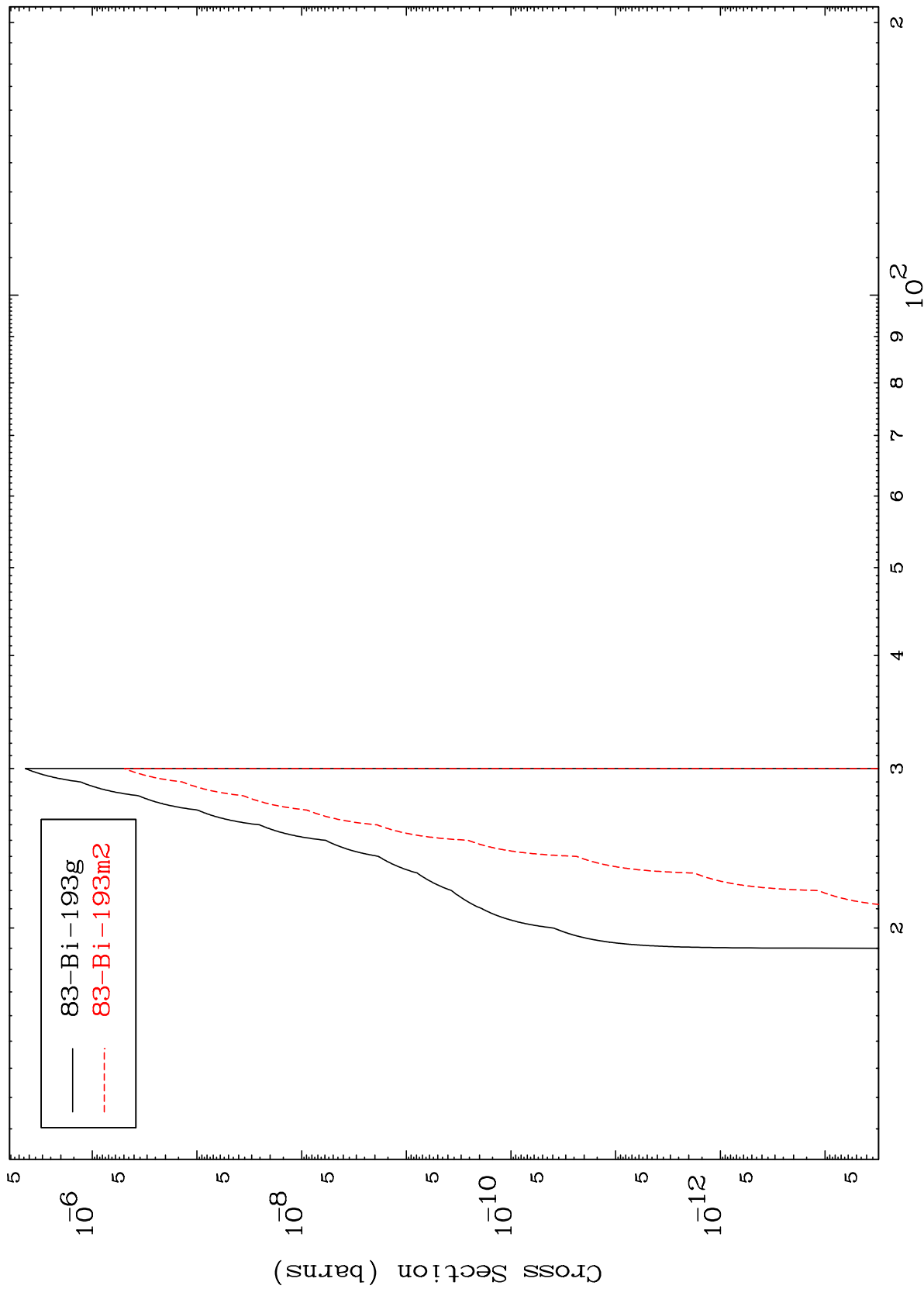


MAT 8513

85-At-199

(n,3n) α

Radionuclide Production Cross Section

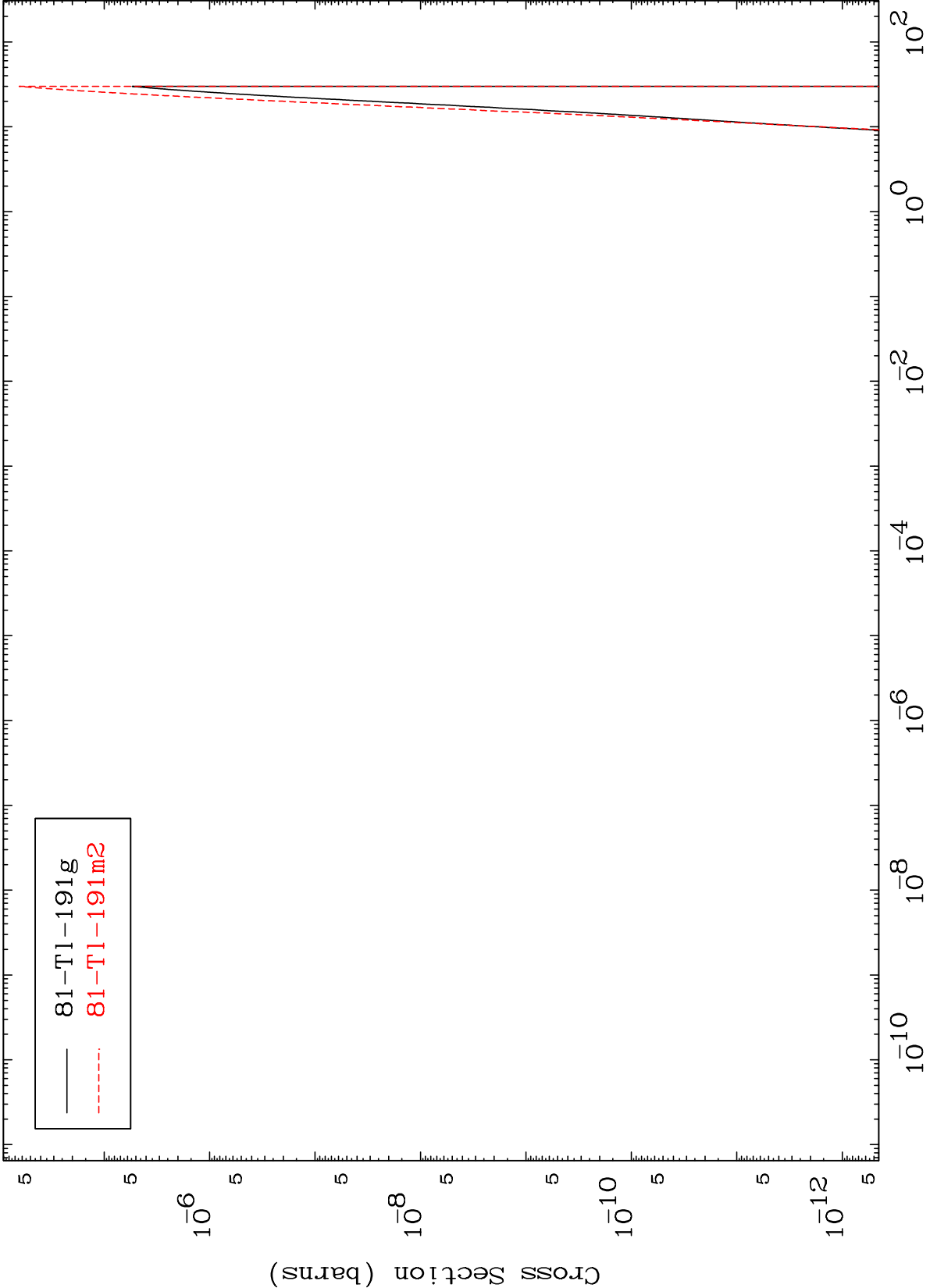


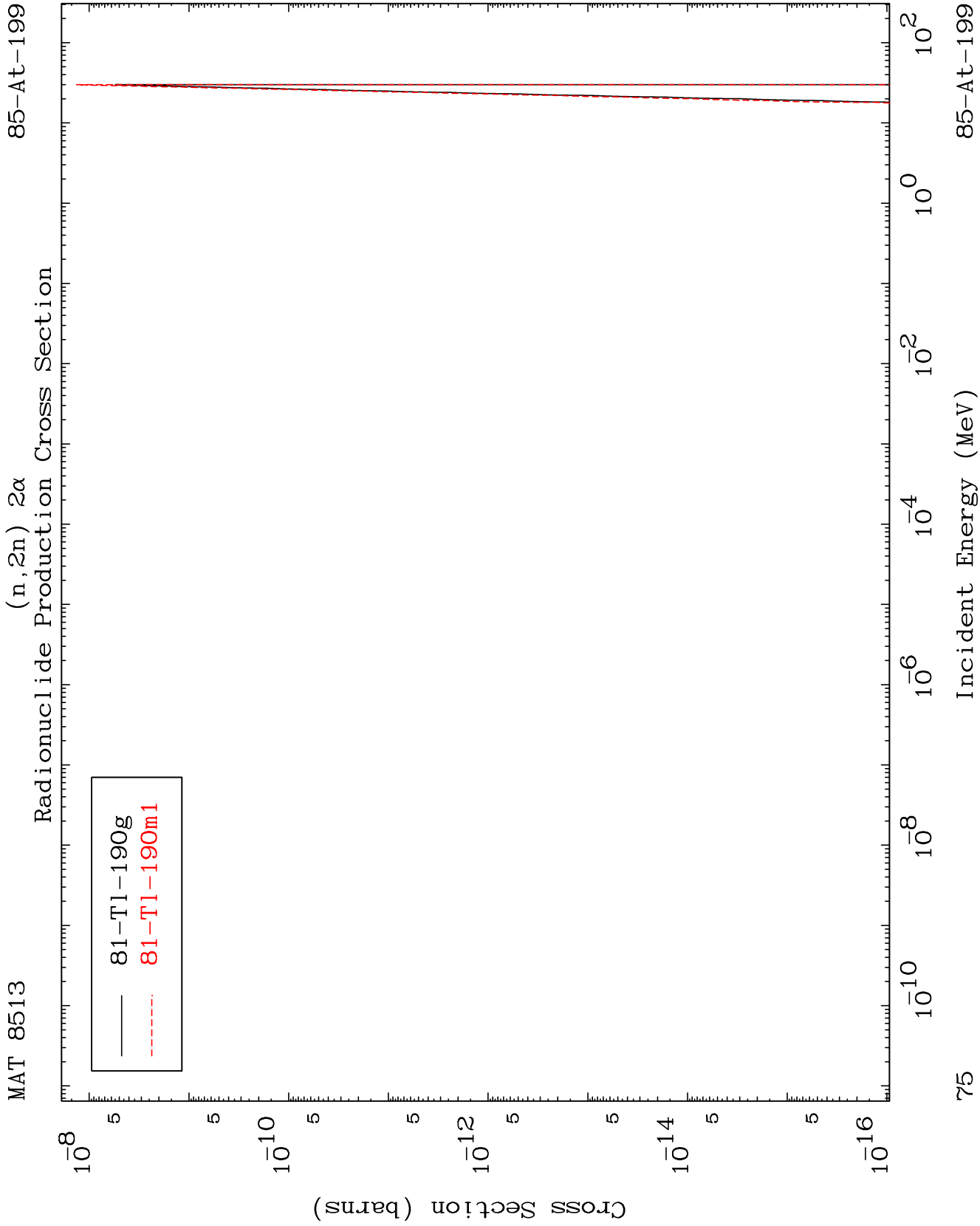
85-At-199

Incident Energy (MeV)

73

Radionuclide Production Cross Section

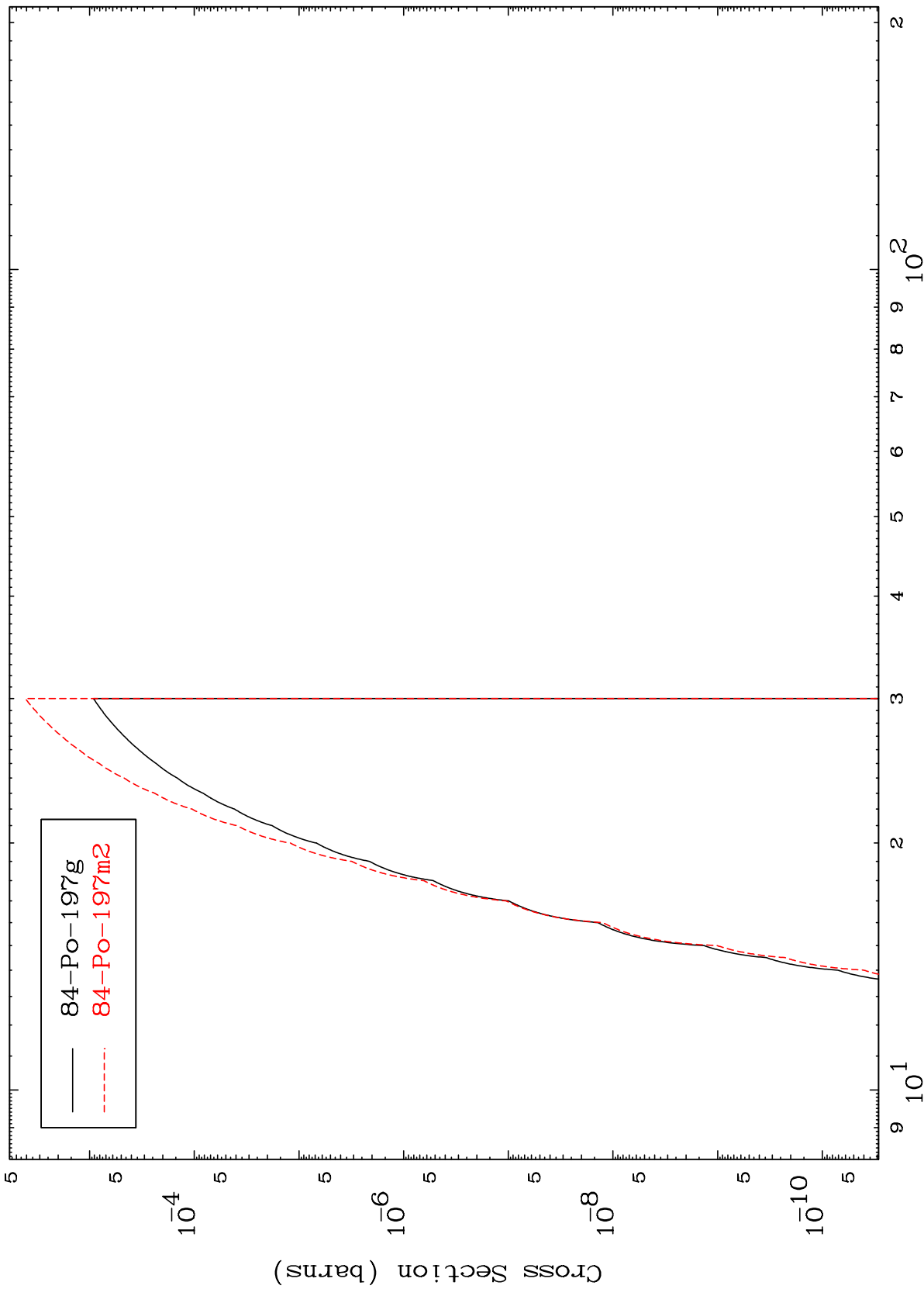




MAT 8513

85-At-199

(n,n') d
Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-199

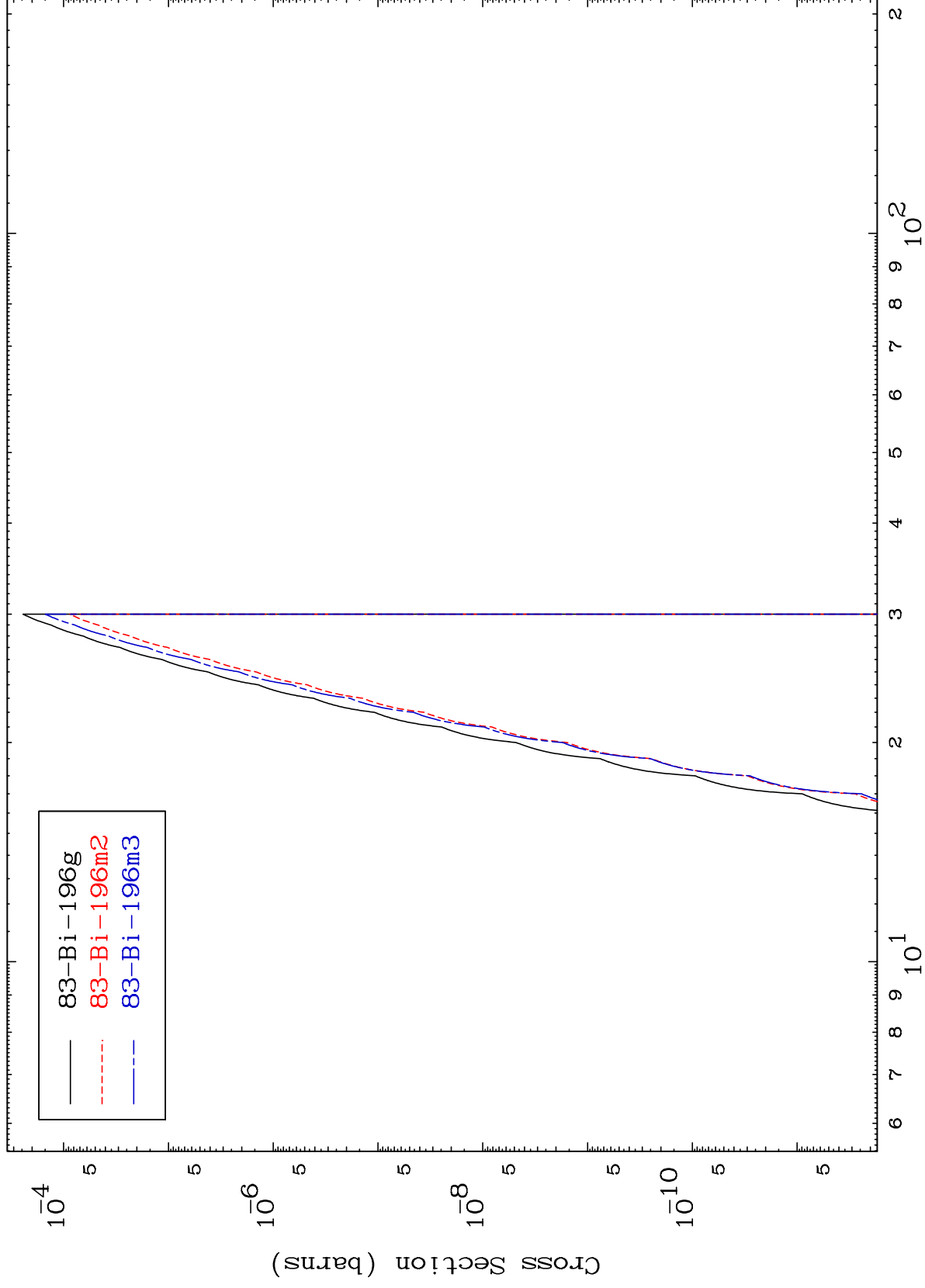
76

MAT 8513

(n,n') He-3

85-At-199

Radionuclide Production Cross Section

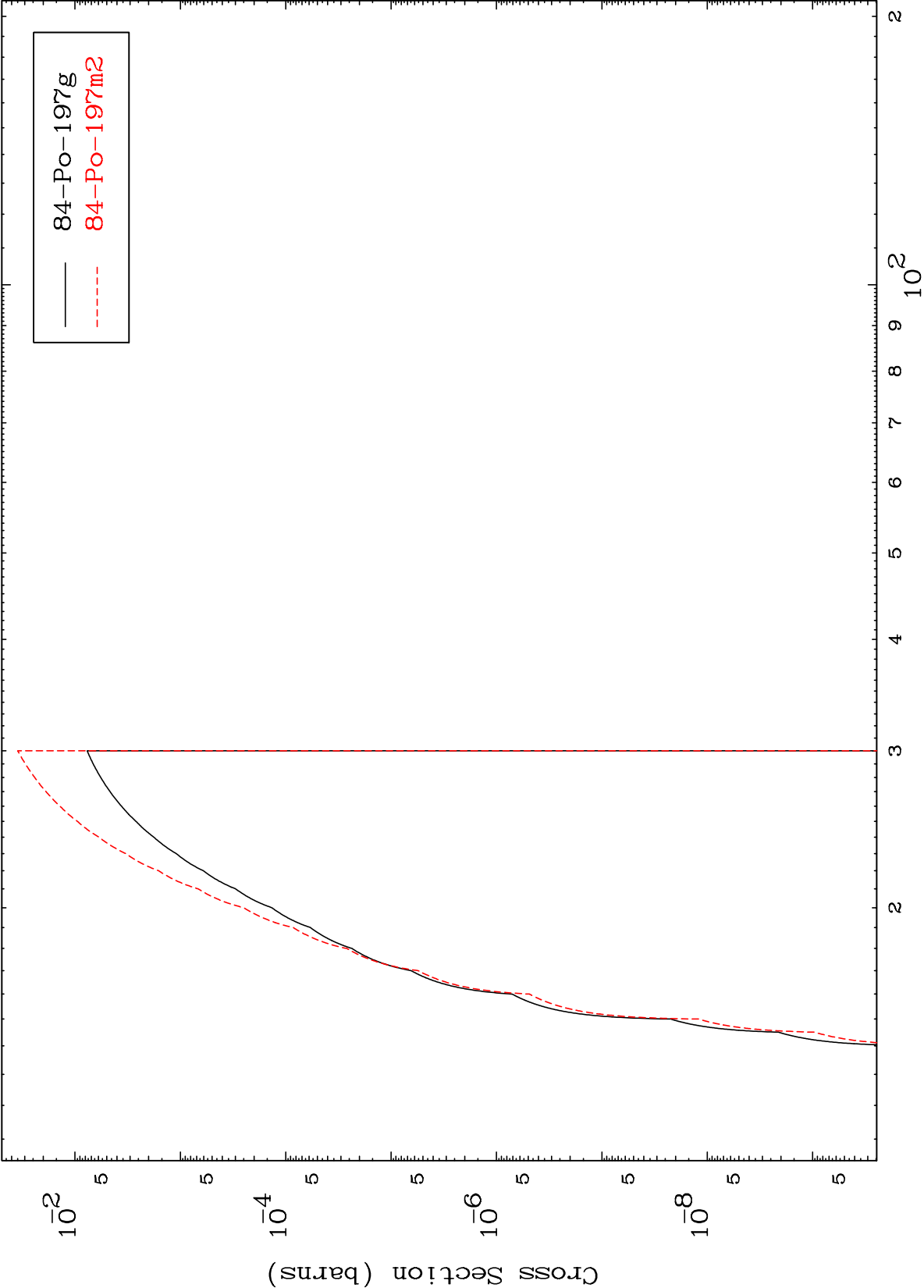


77

Incident Energy (MeV)

85-At-199

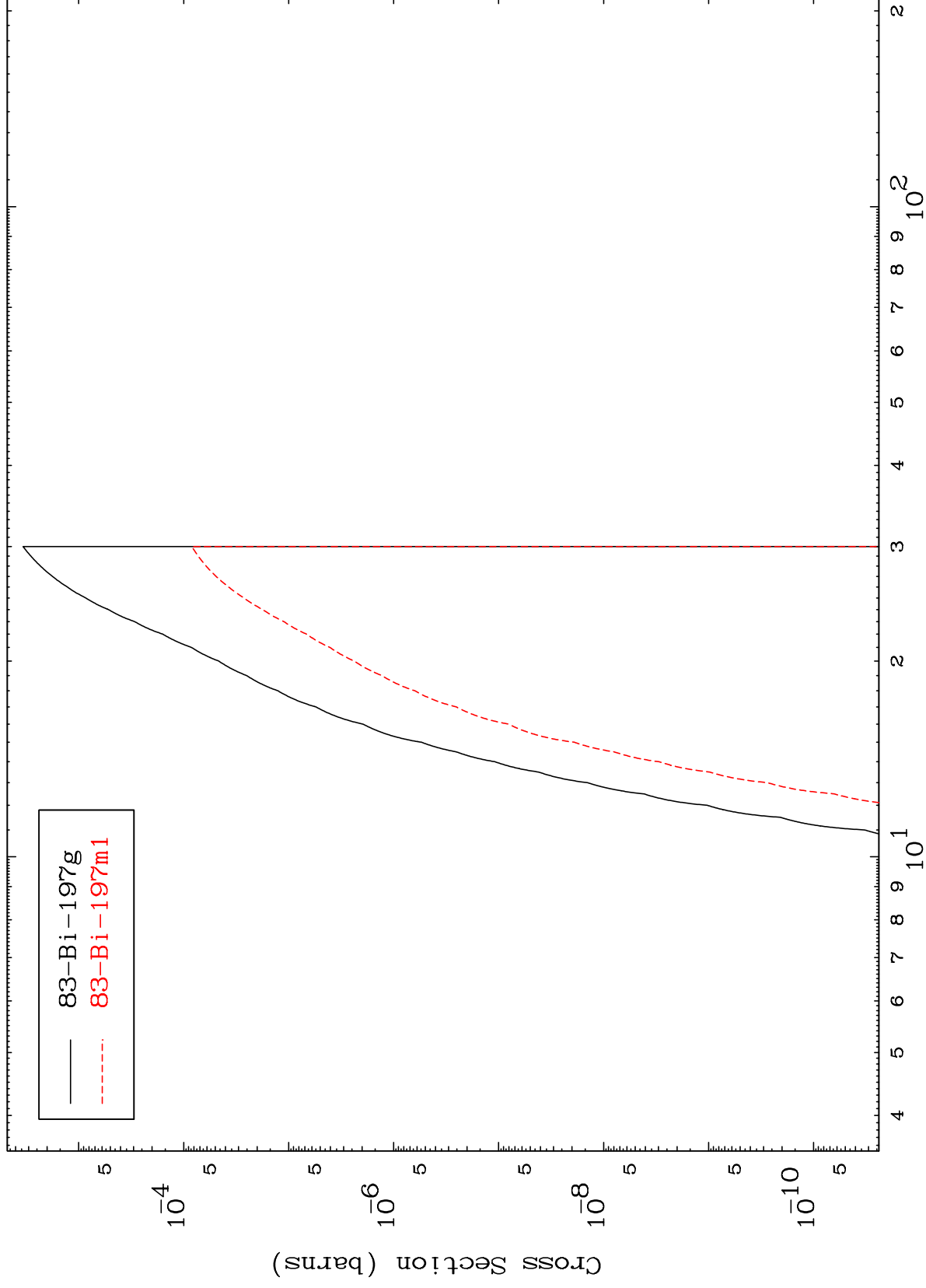
(n,2n) p
Radionuclide Production Cross Section



MAT 8513

85-At-199

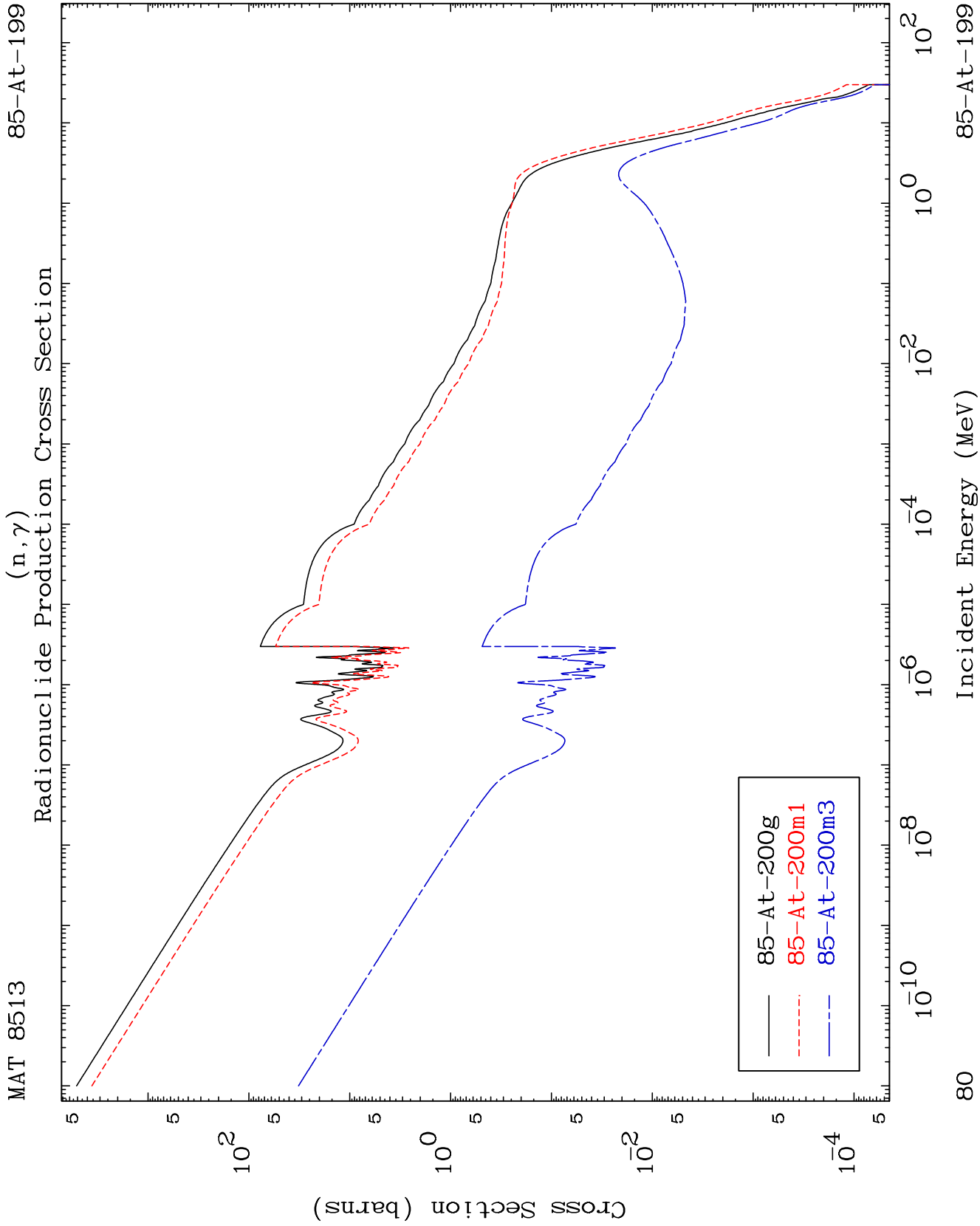
Radionuclide Production Cross Section
(n,2n) p

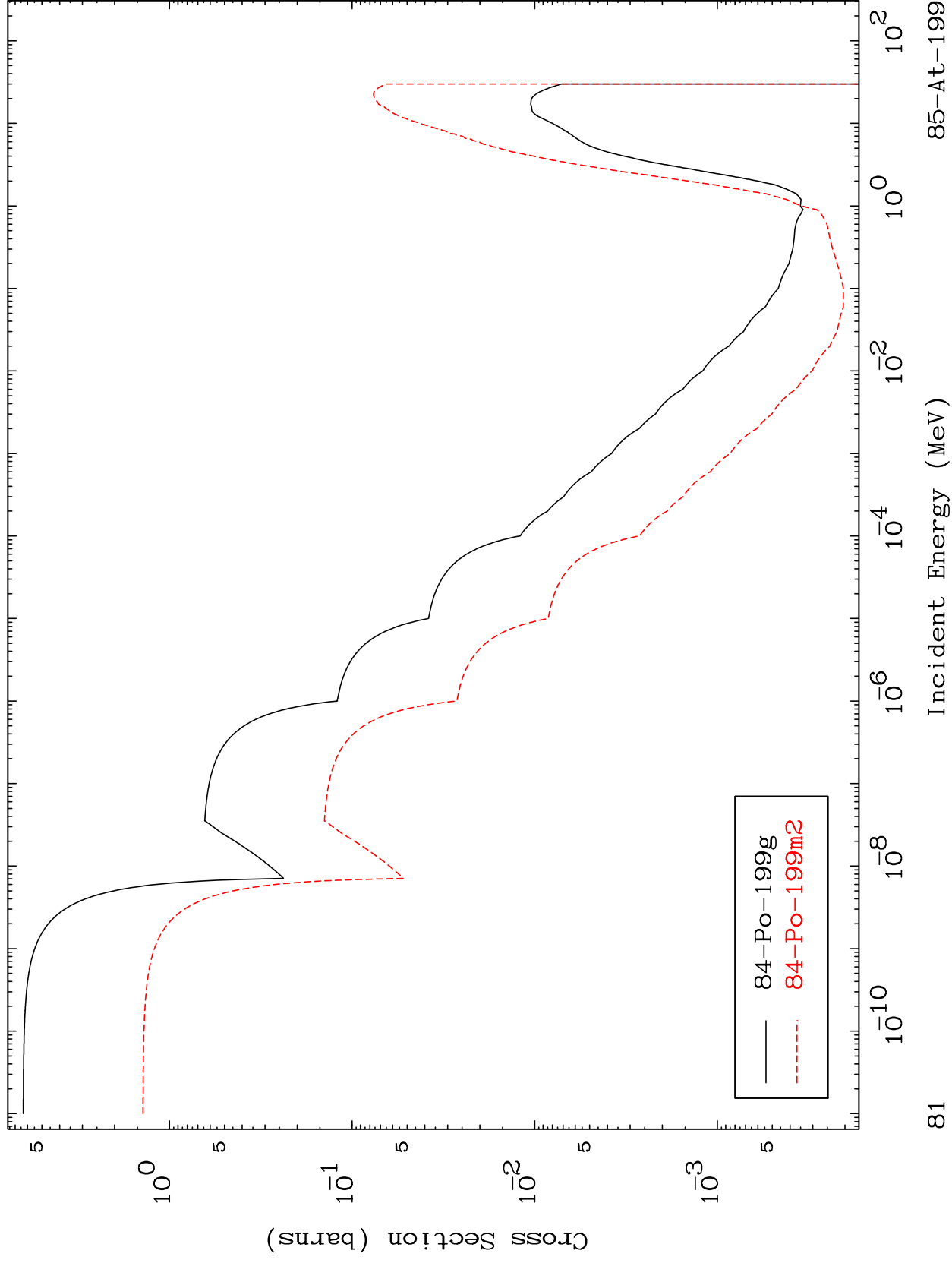


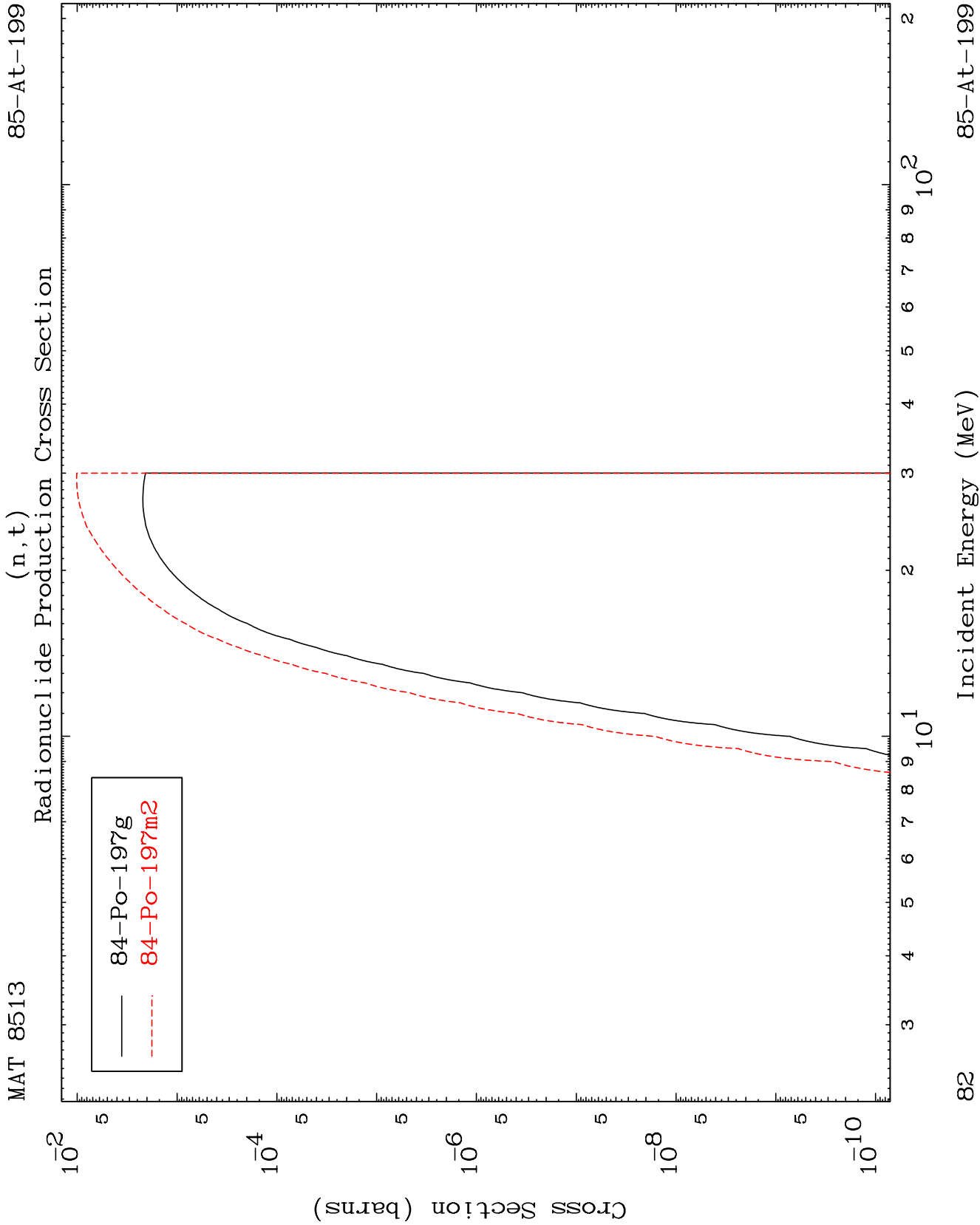
79

Incident Energy (MeV)

85-At-199



(n,p)
Radionuclide Production Cross Section

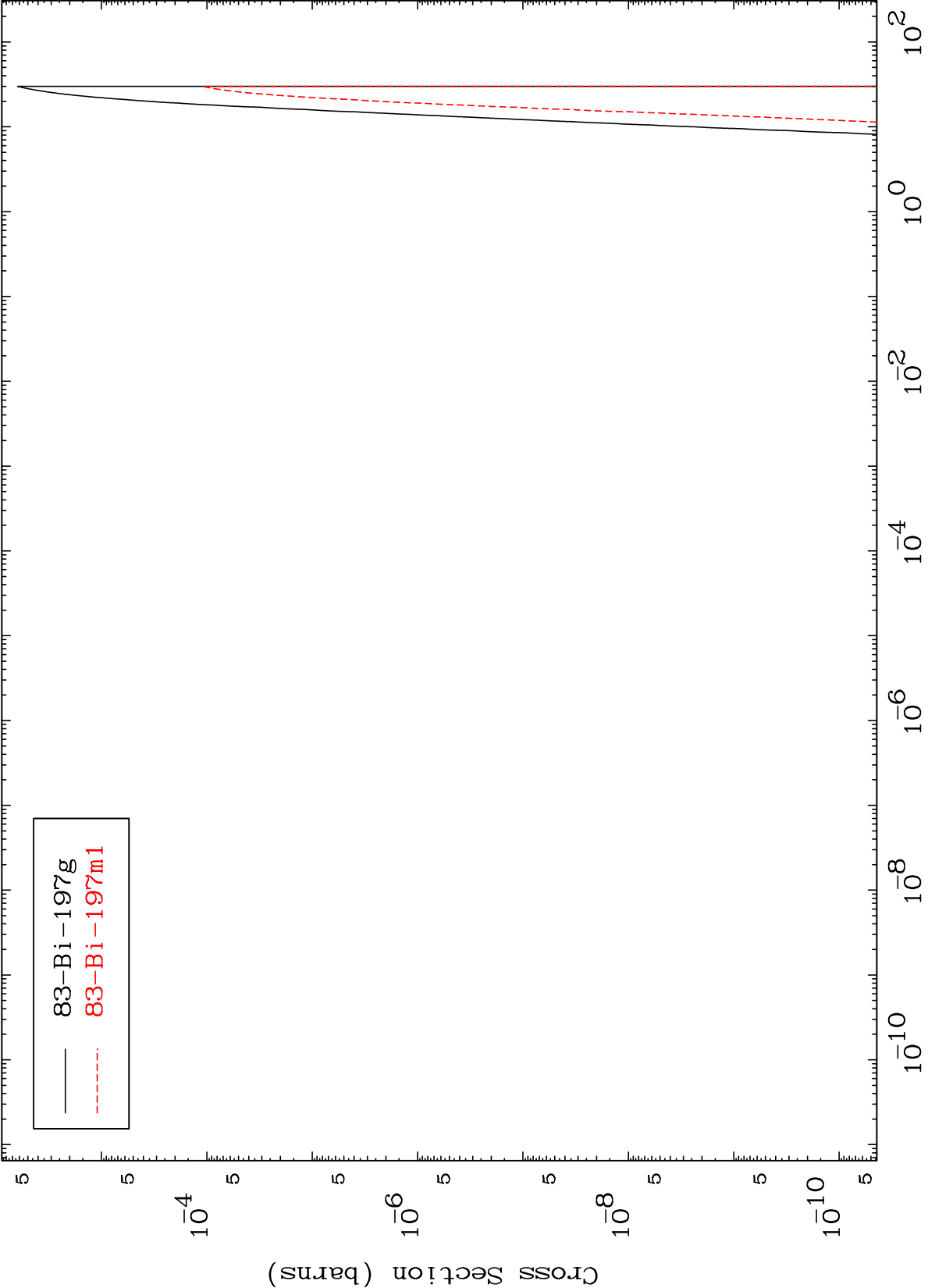


MAT 8513

(n,He-3)

85-At-199

Radionuclide Production Cross Section

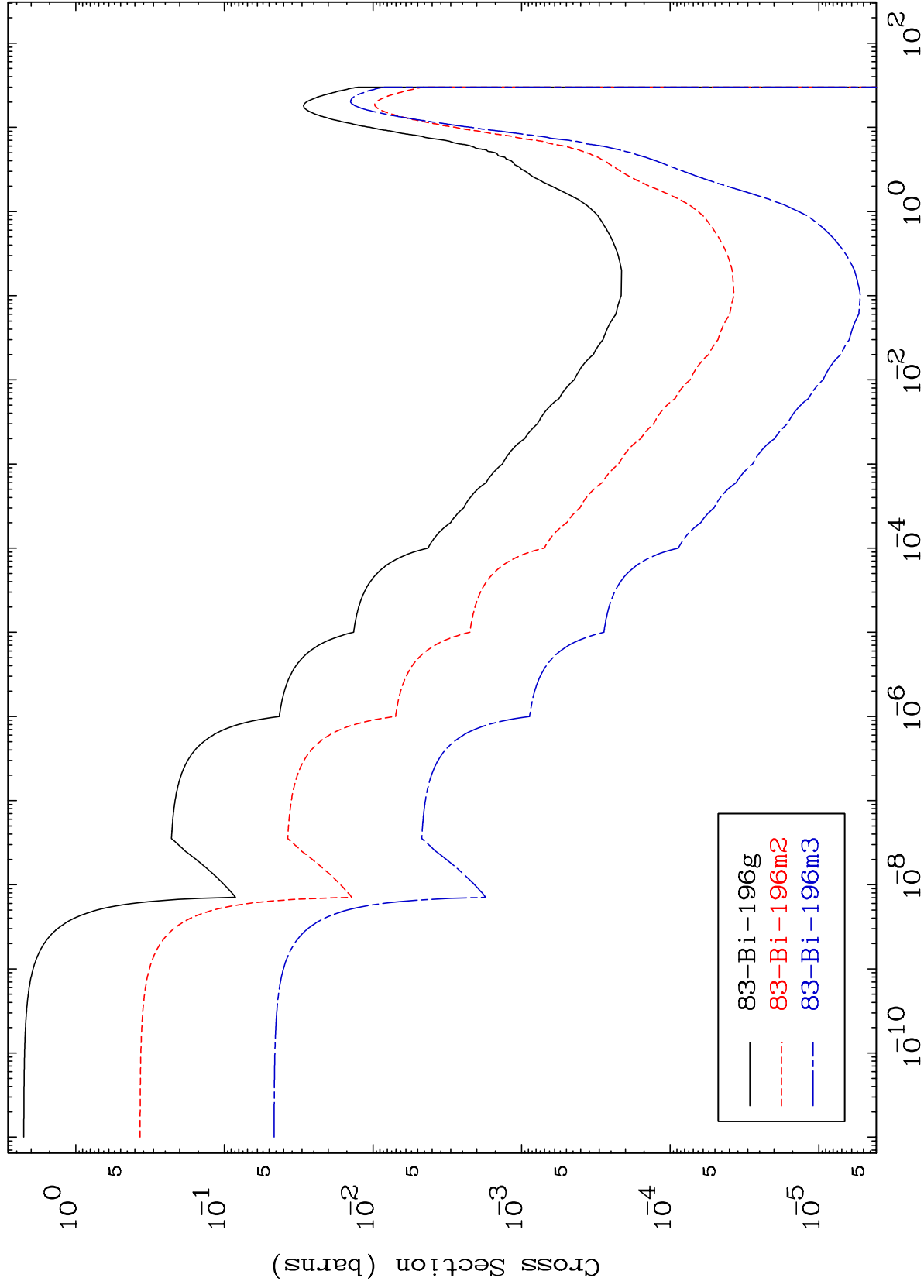


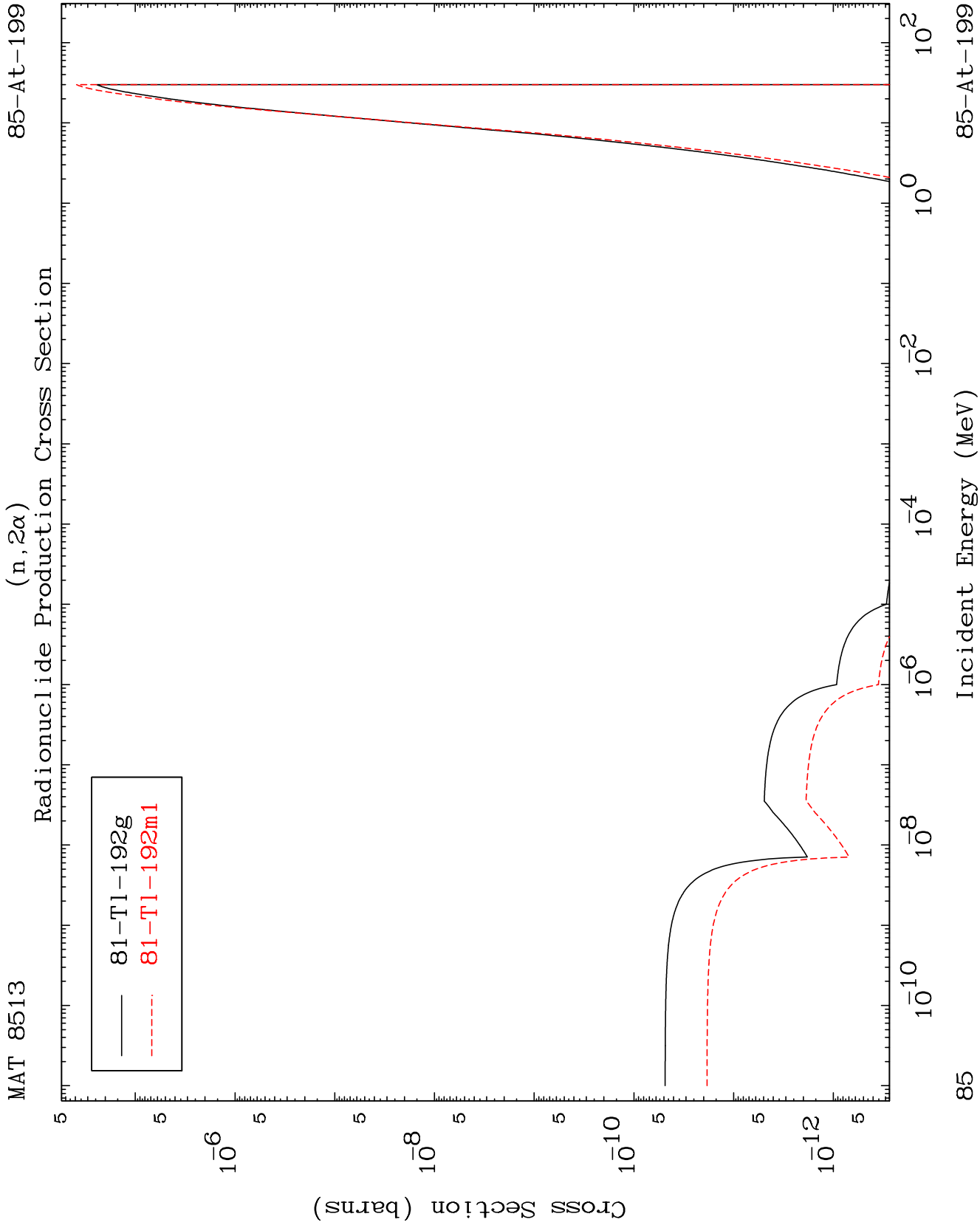
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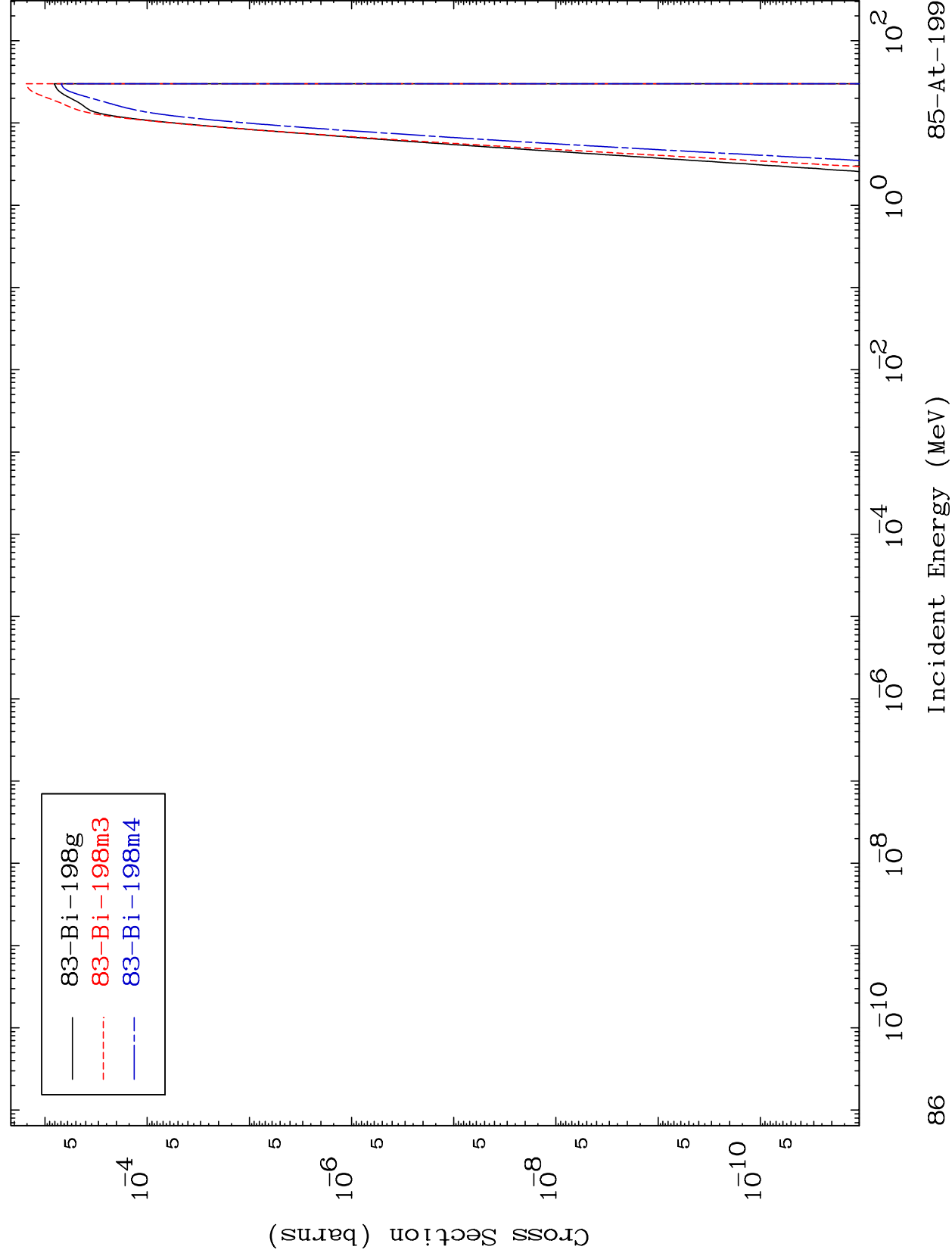
Incident Energy (MeV)

85-At-199

Radionuclide Production Cross Section (n, α)





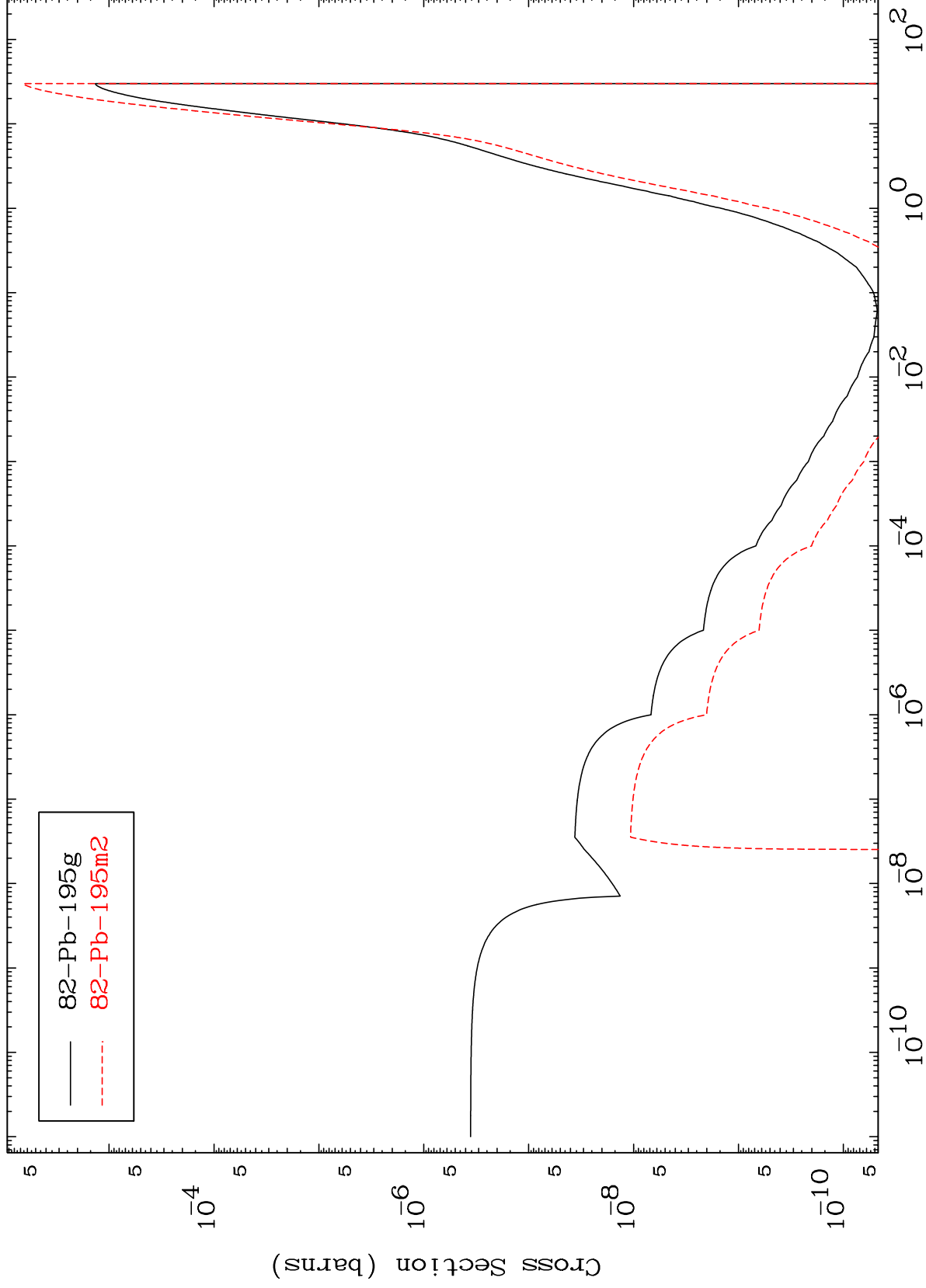
(n,2p)
Radionuclide Production Cross Section

MAT 8513

(n,p) α

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Radionuclide Production Cross Section



87

Incident Energy (MeV)

85-At-199

Radionuclide Production Cross Section

